

Program EVALPLOT
(Version 2021-1)

by

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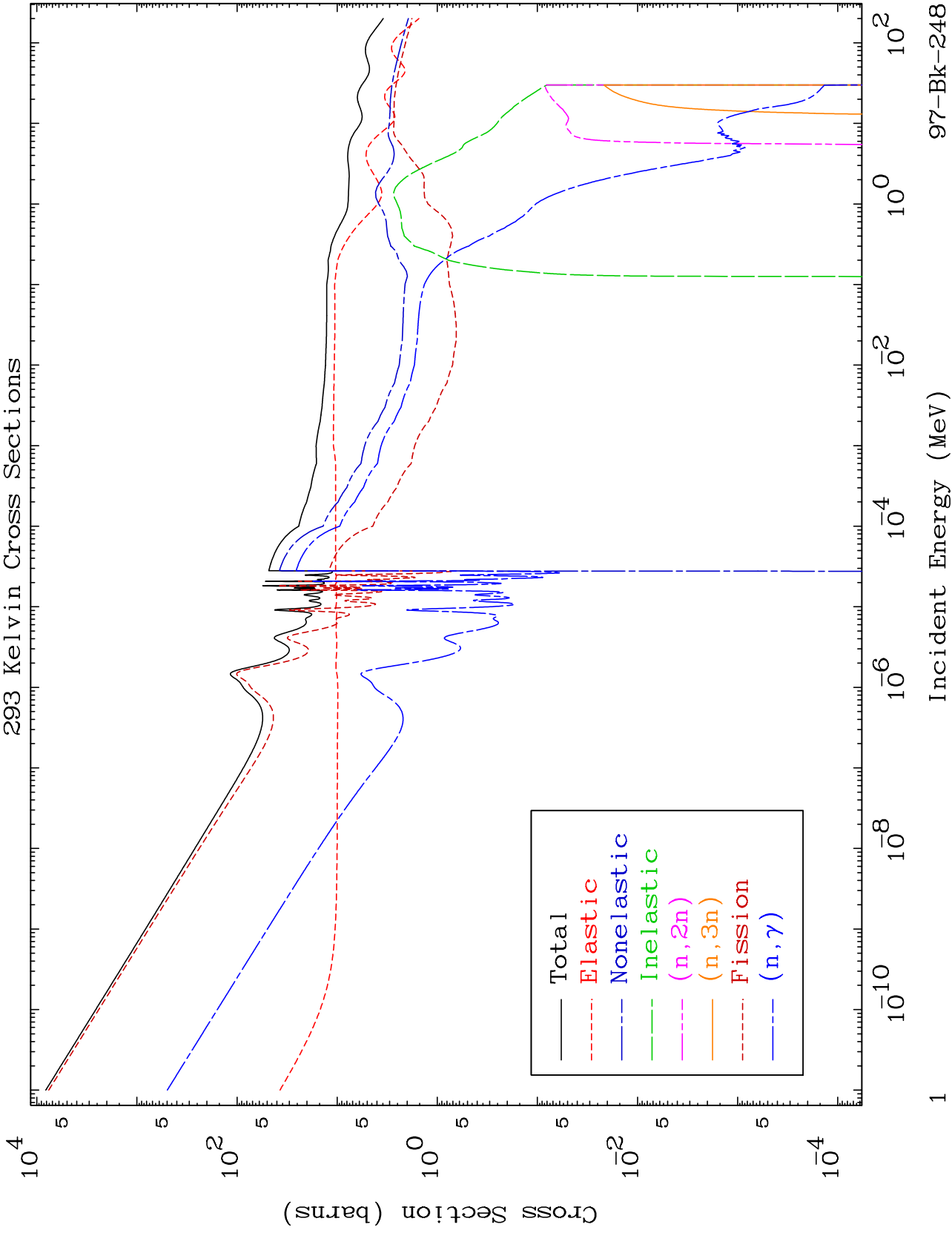
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Press Mouse Button to Start

MAT 9749

Neutron Major
293 Kelvin Cross Sections

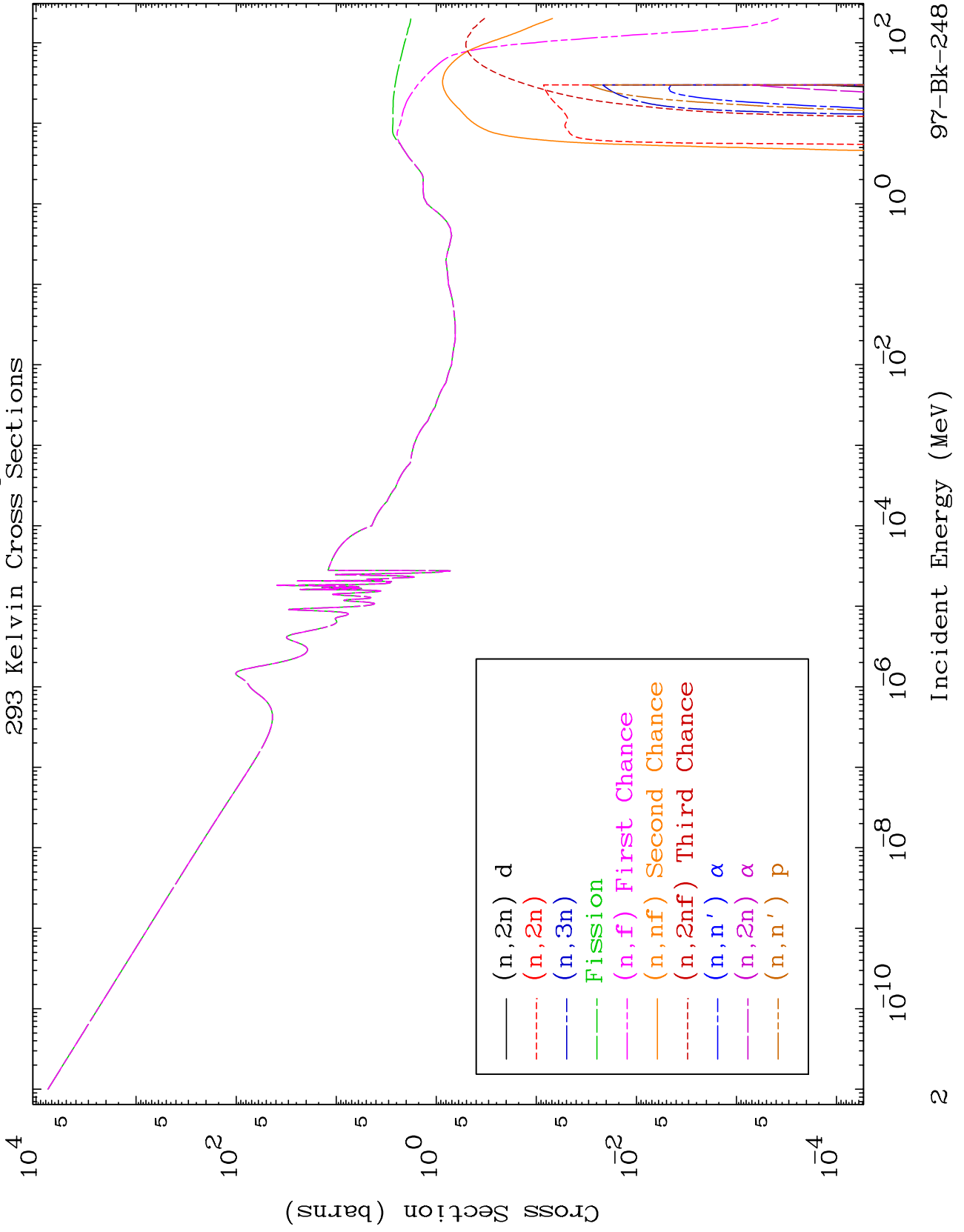
97-Bk-248



MAT 9749

Neutron Absorption
293 Kelvin Cross Sections

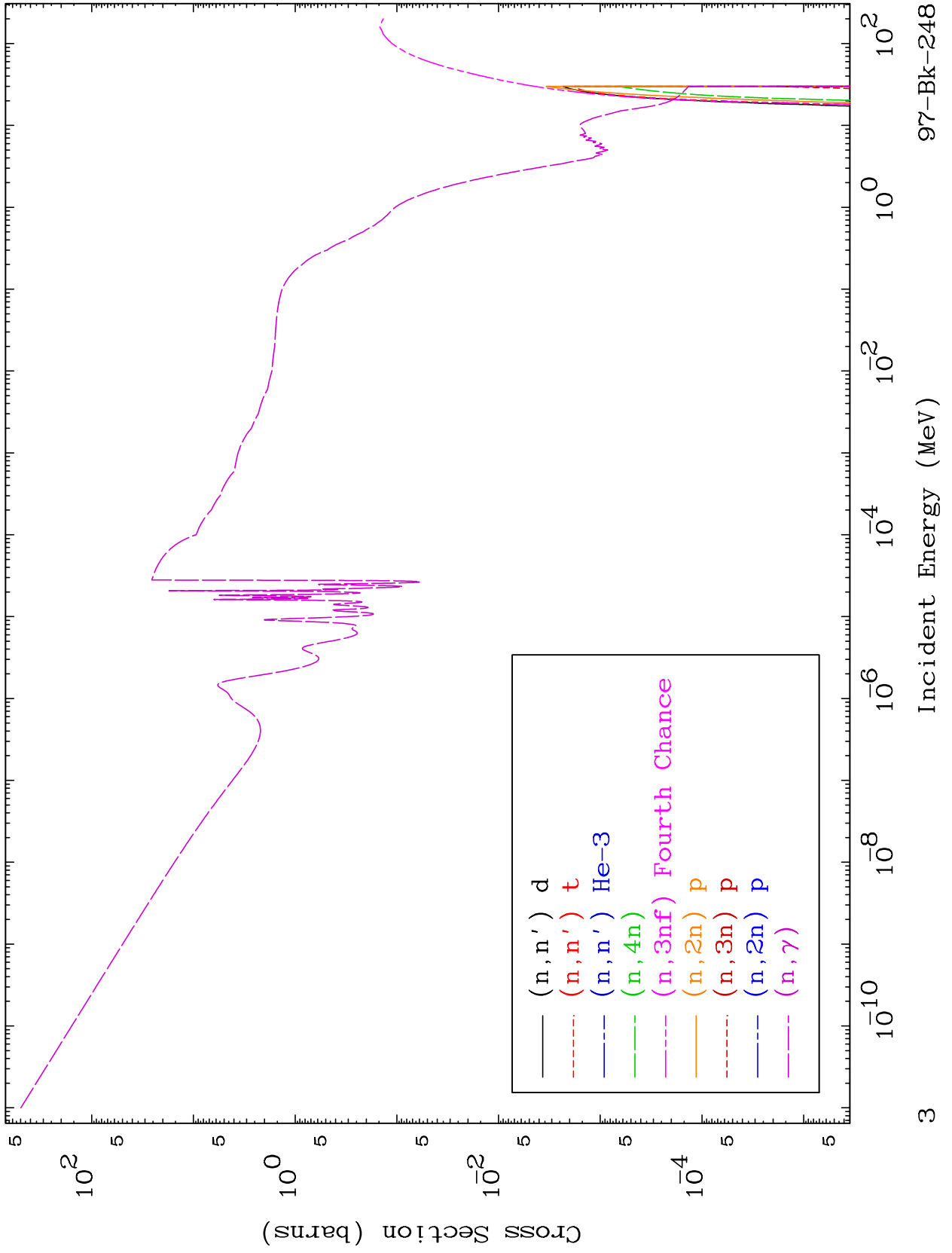
97-Bk-248



MAT 9749

Neutron Absorption
293 Kelvin Cross Sections

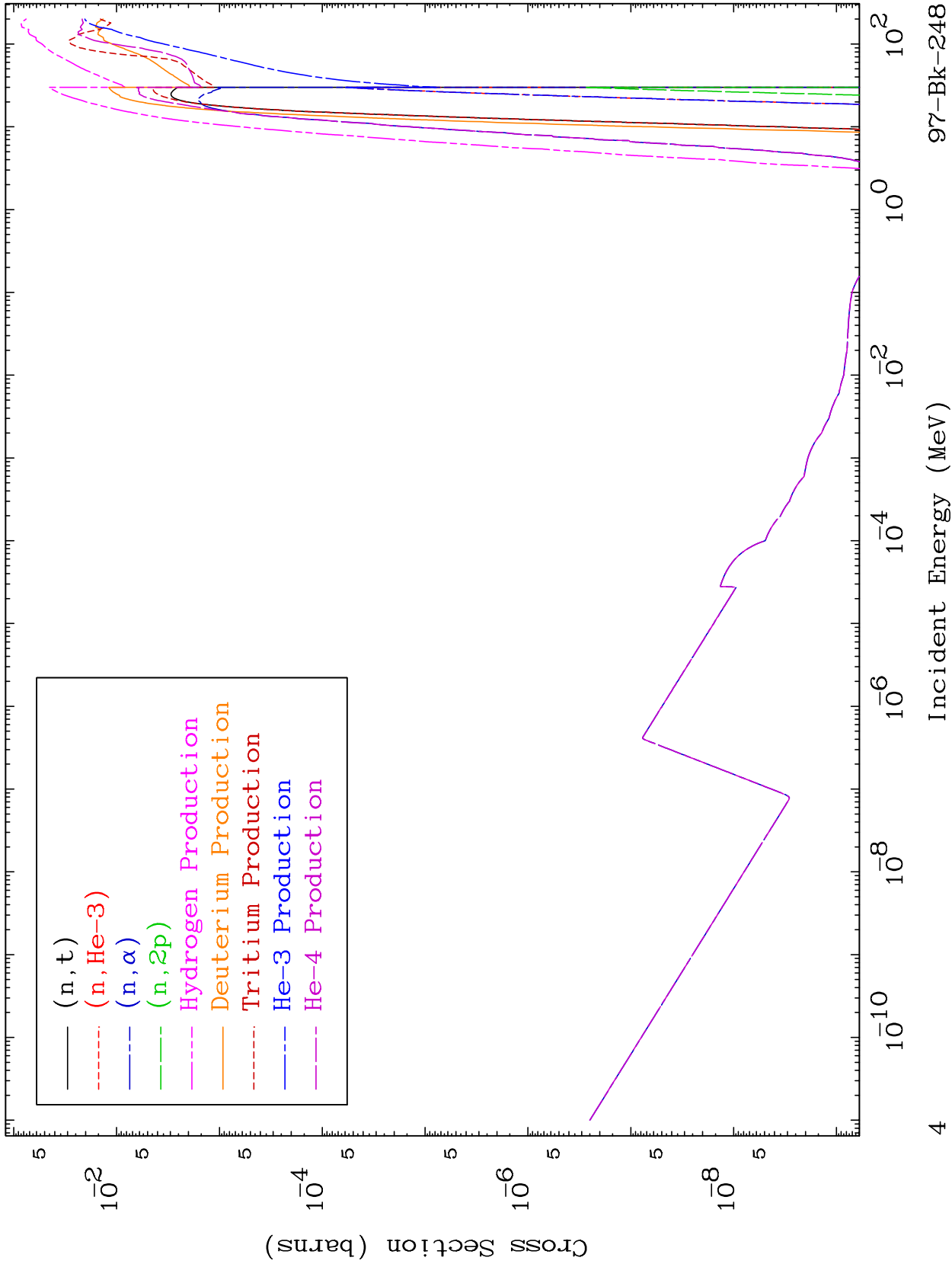
97-Bk-248



MAT 9749

Neutron Absorption
293 Kelvin Cross Sections

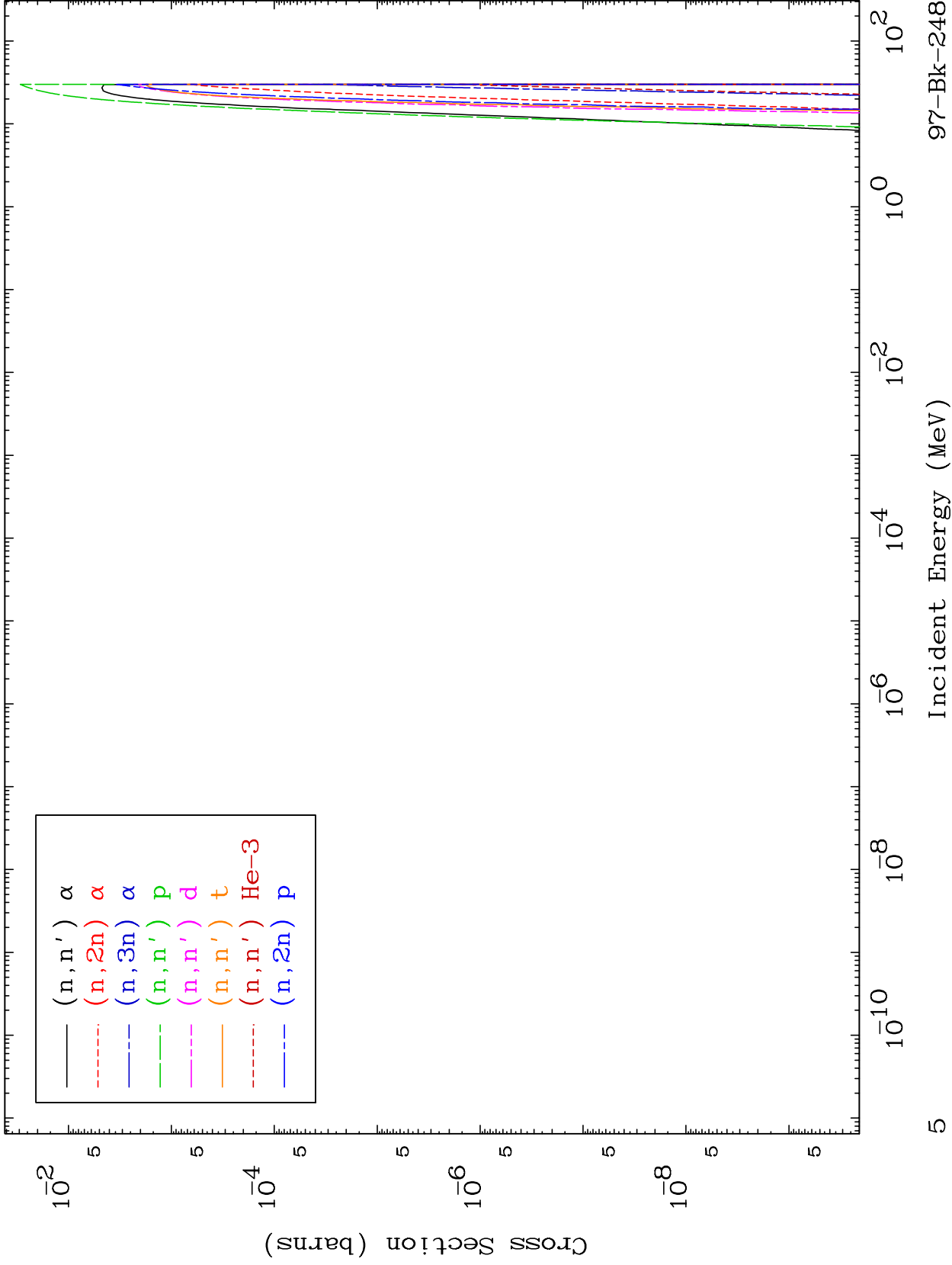
97-Bk-248



MAT 9749

Charged Particle
293 Kelvin Cross Sections

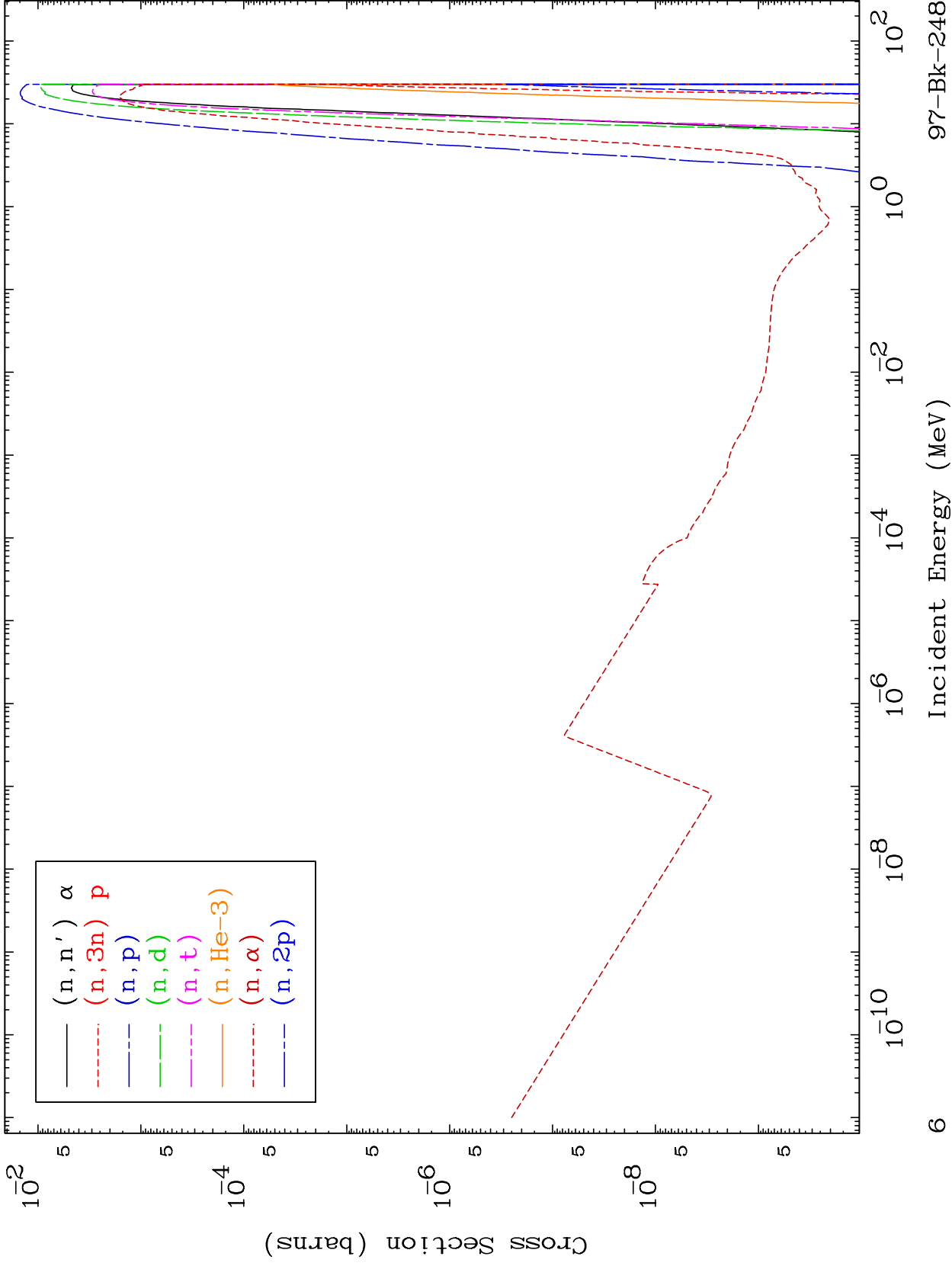
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MAT 9749

Charged Particle
293 Kelvin Cross Sections

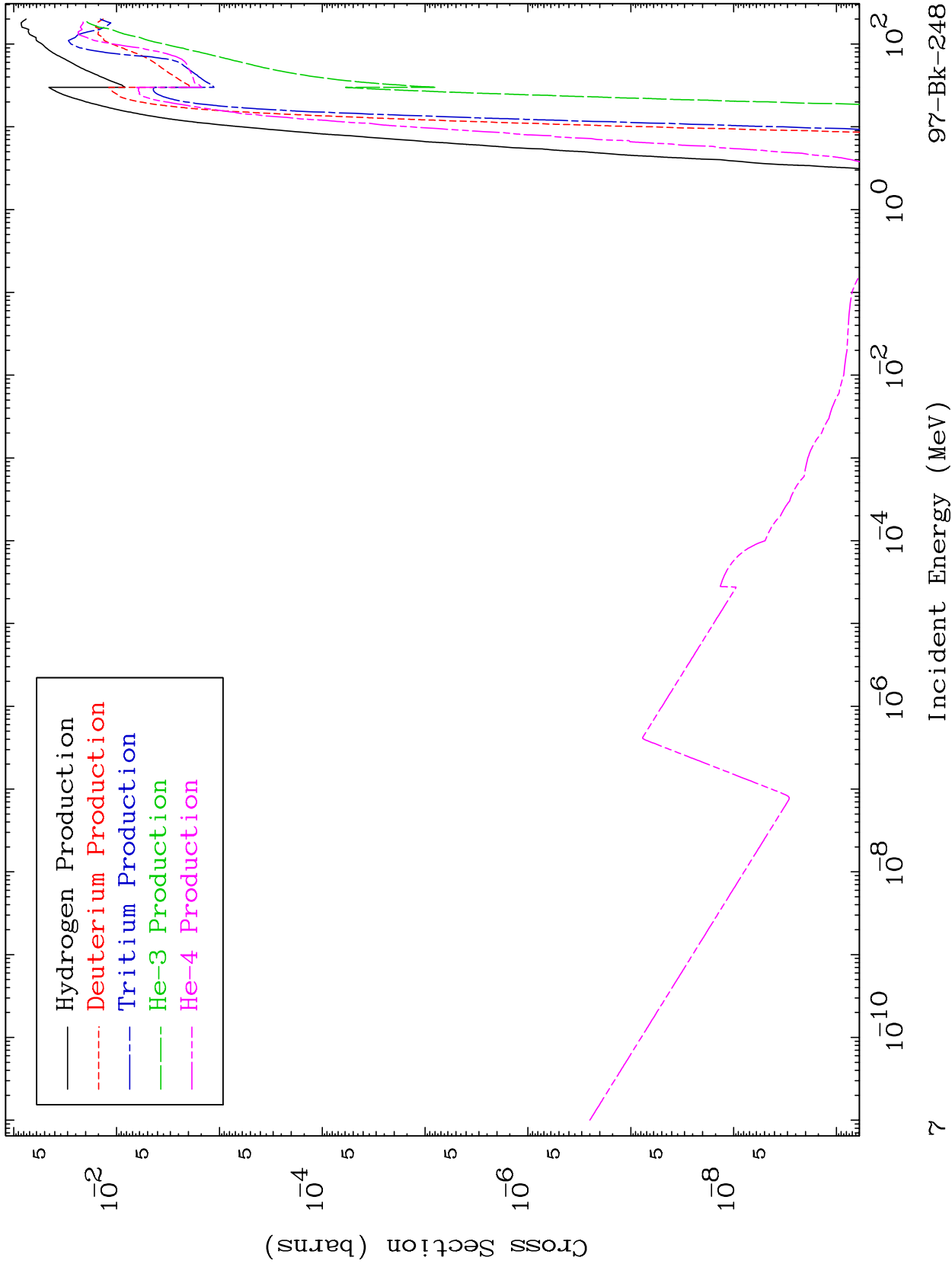
97-Bk-248



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Particle Production
293 Kelvin Cross Sections

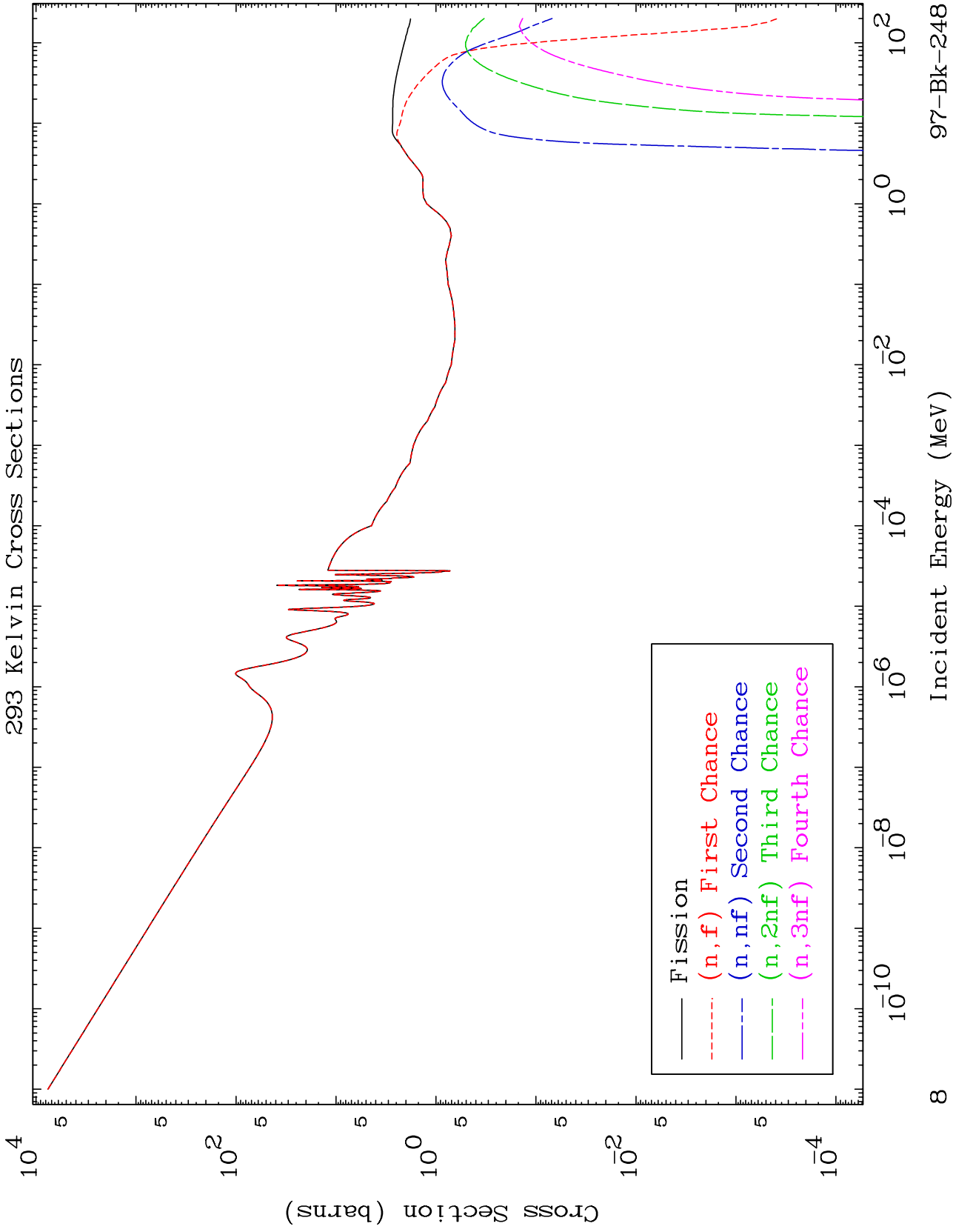
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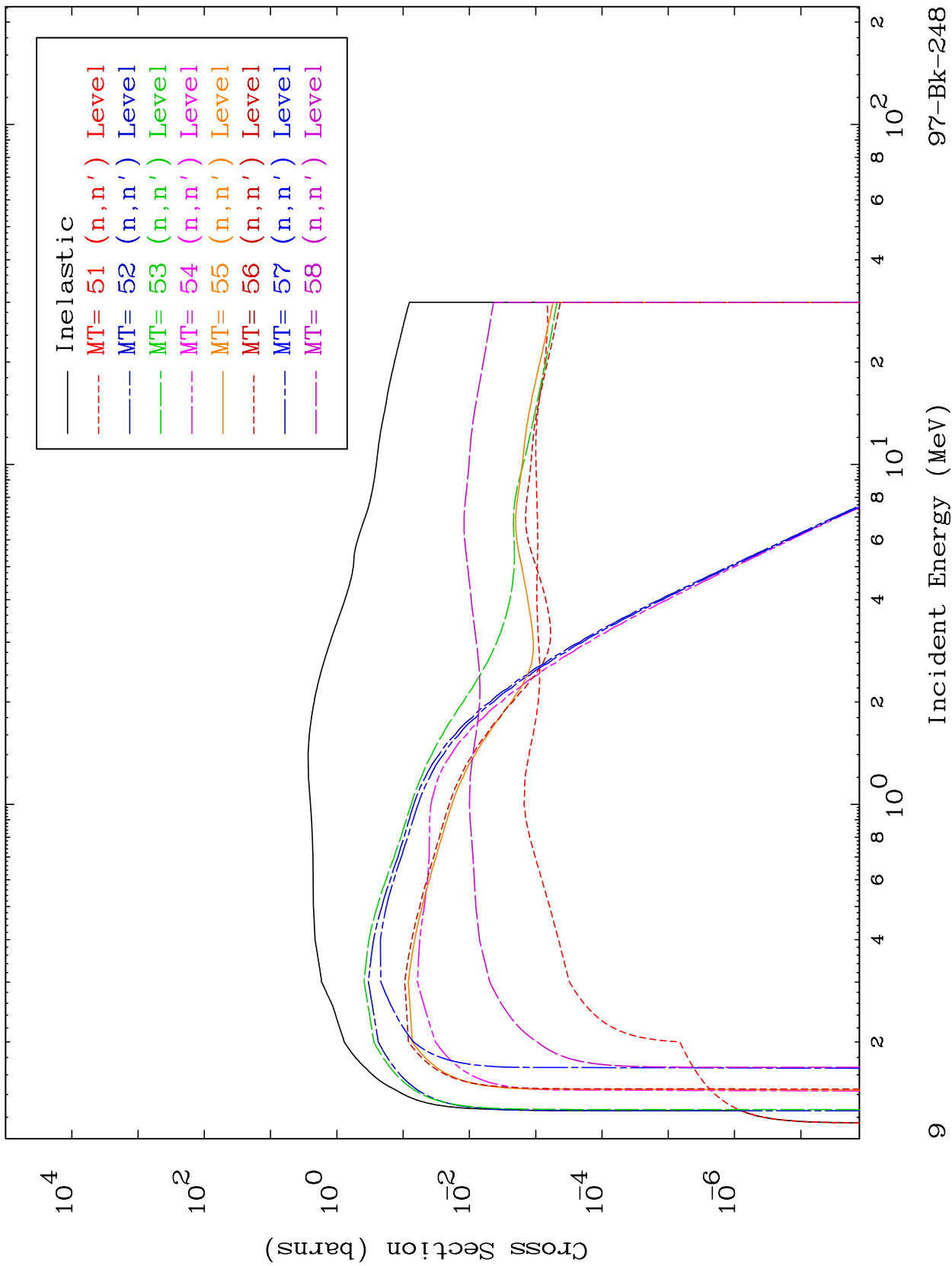


MAT 9749

Fission
293 Kelvin Cross Sections

97-Bk-248

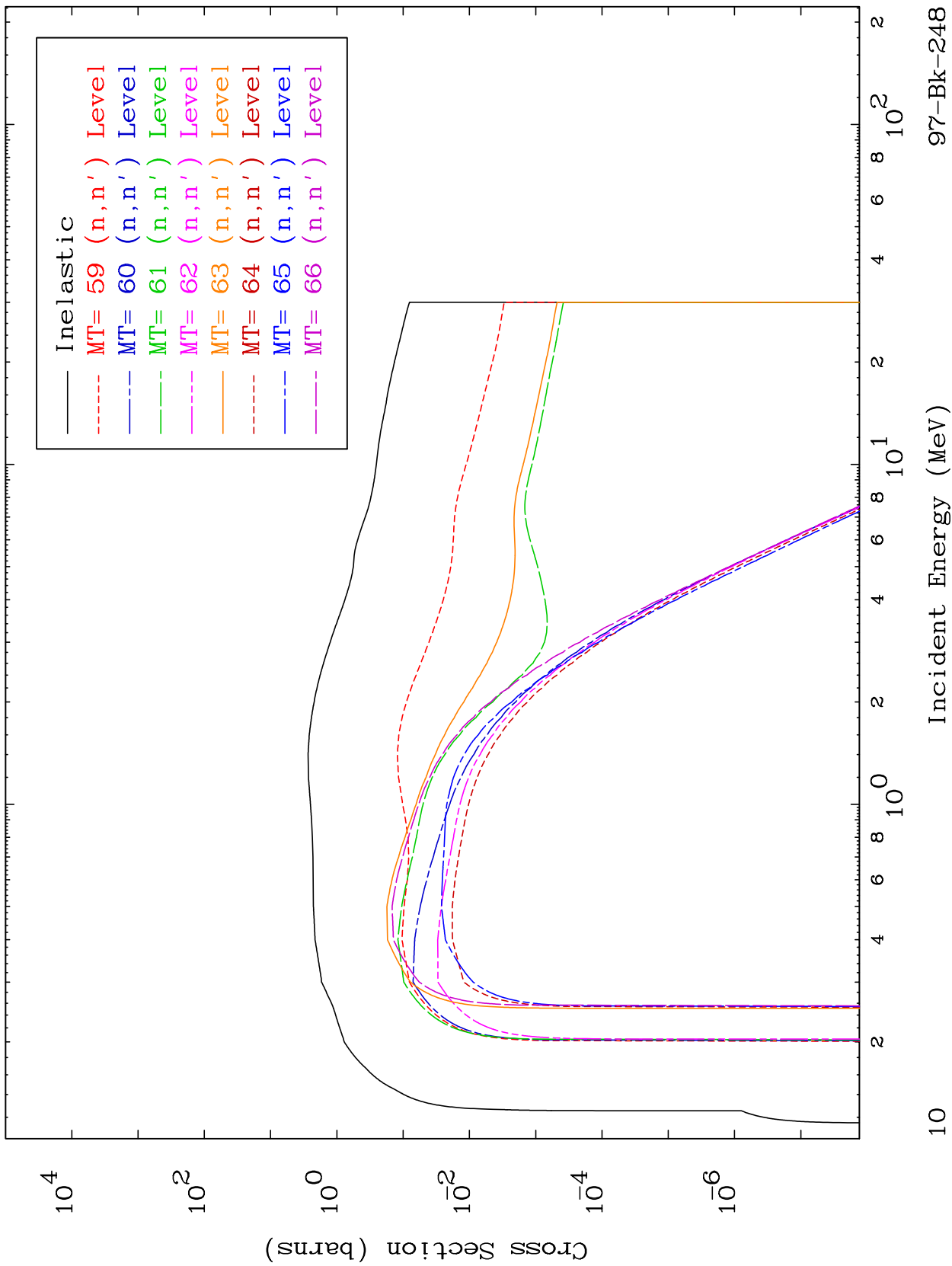




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(n,n') Levels
293 Kelvin Cross Sections

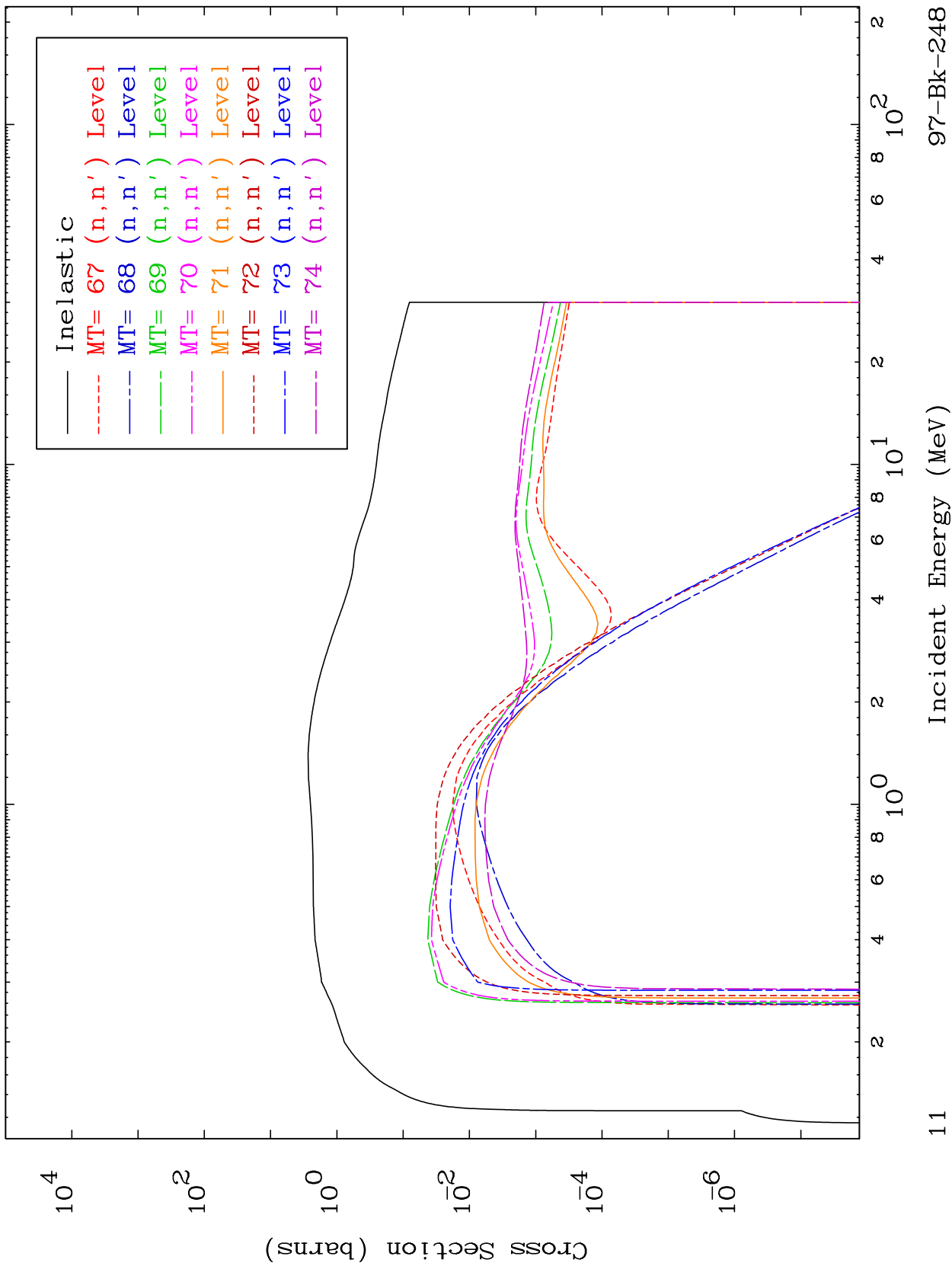
97-Bk-248



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Incident Energy (MeV)

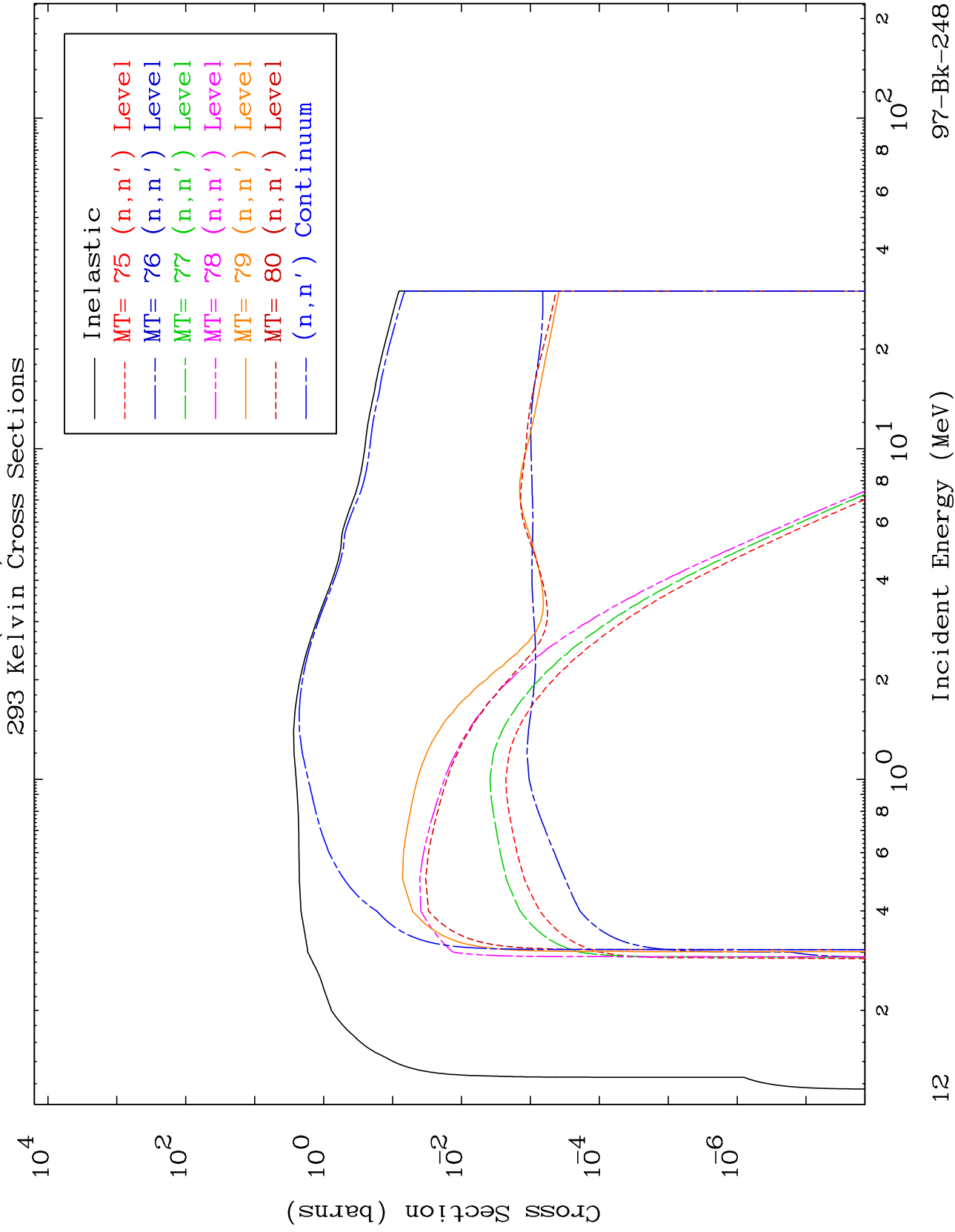
97-Bk-248



MAT 9749

(n,n') Levels

97-Bk-248

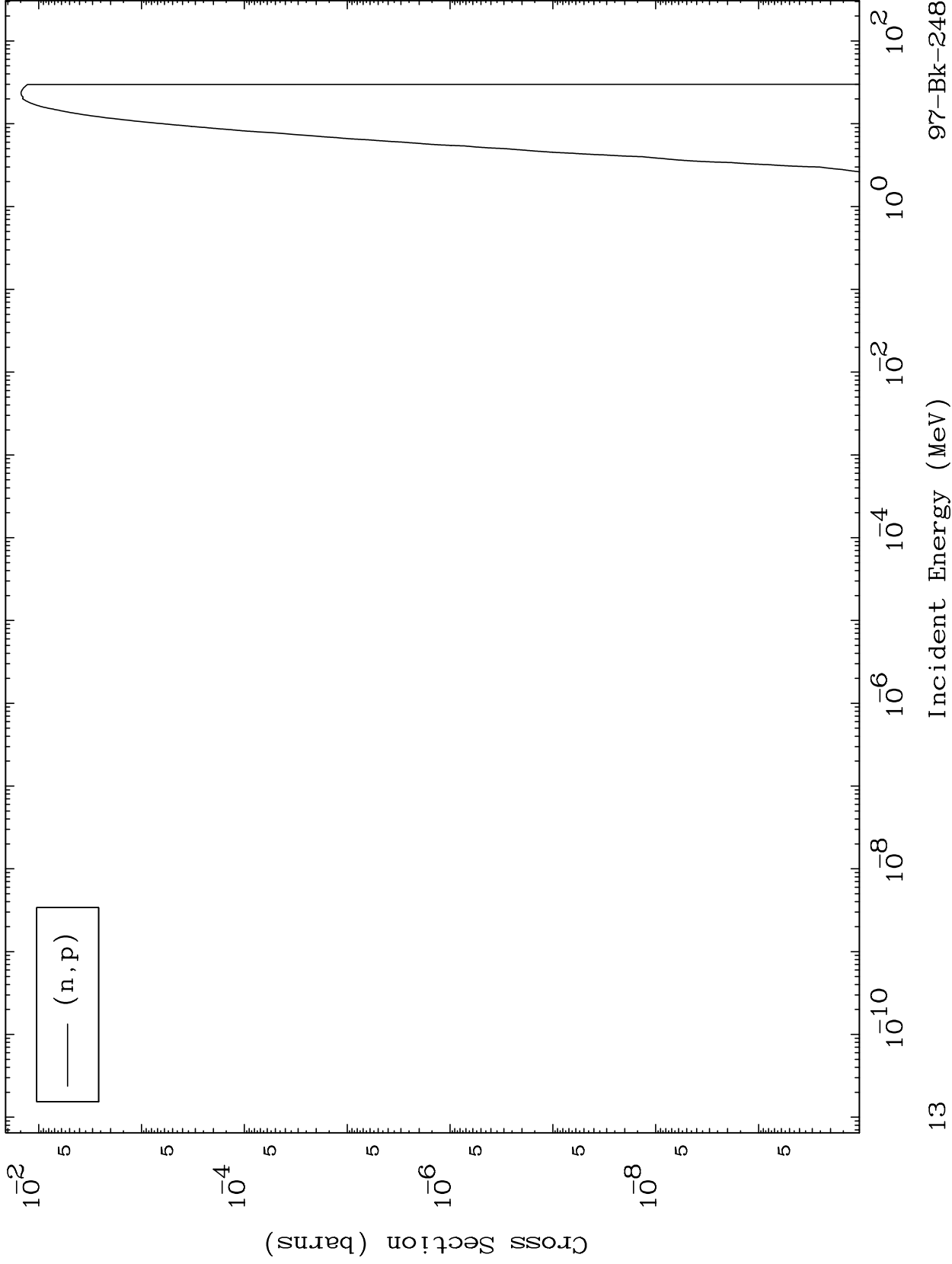


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MAT 9749

(n,p) Levels
293 Kelvin Cross Sections

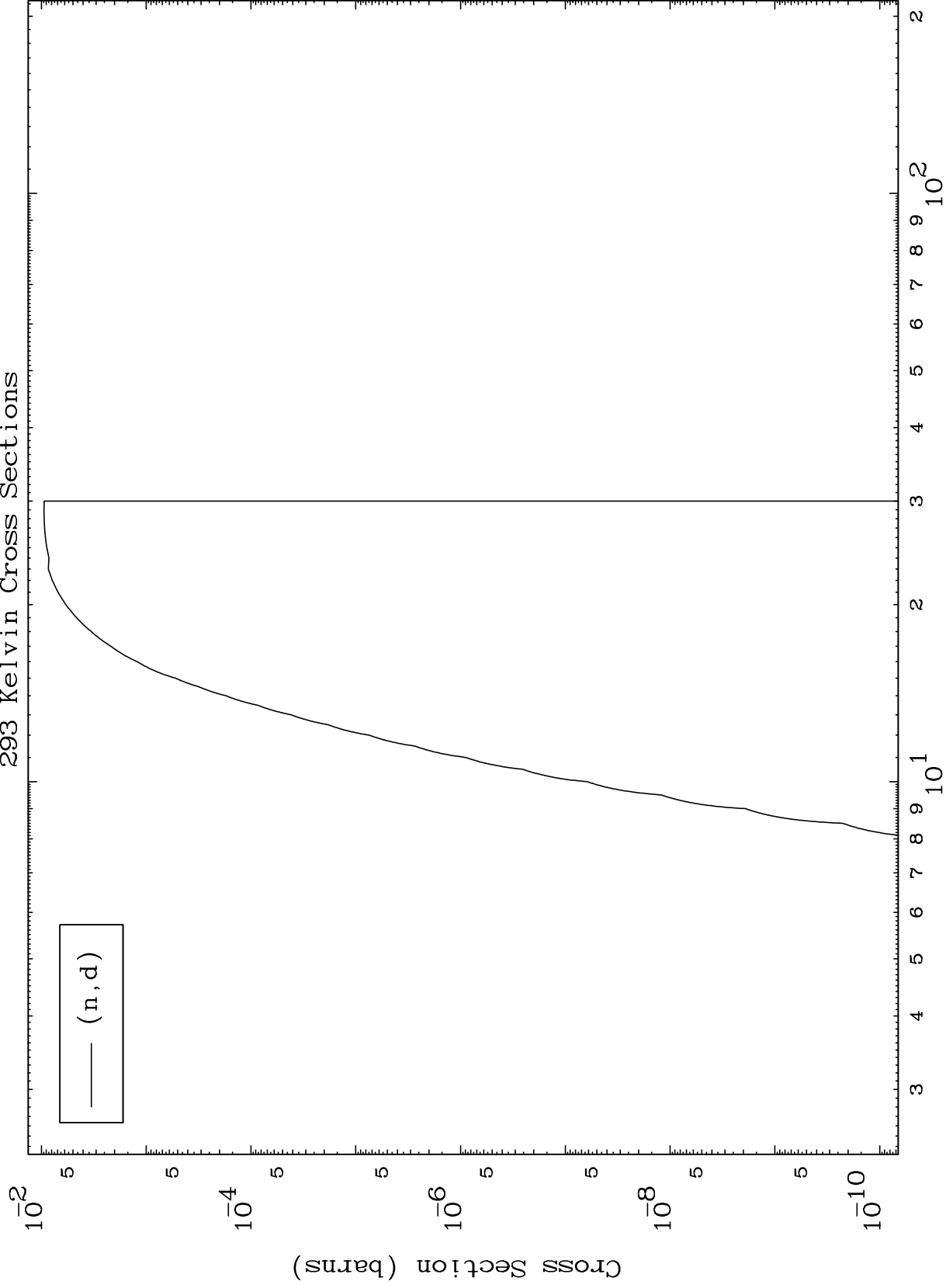
97-Bk-248



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(n,d) Levels
293 Kelvin Cross Sections

97-Bk-248



14

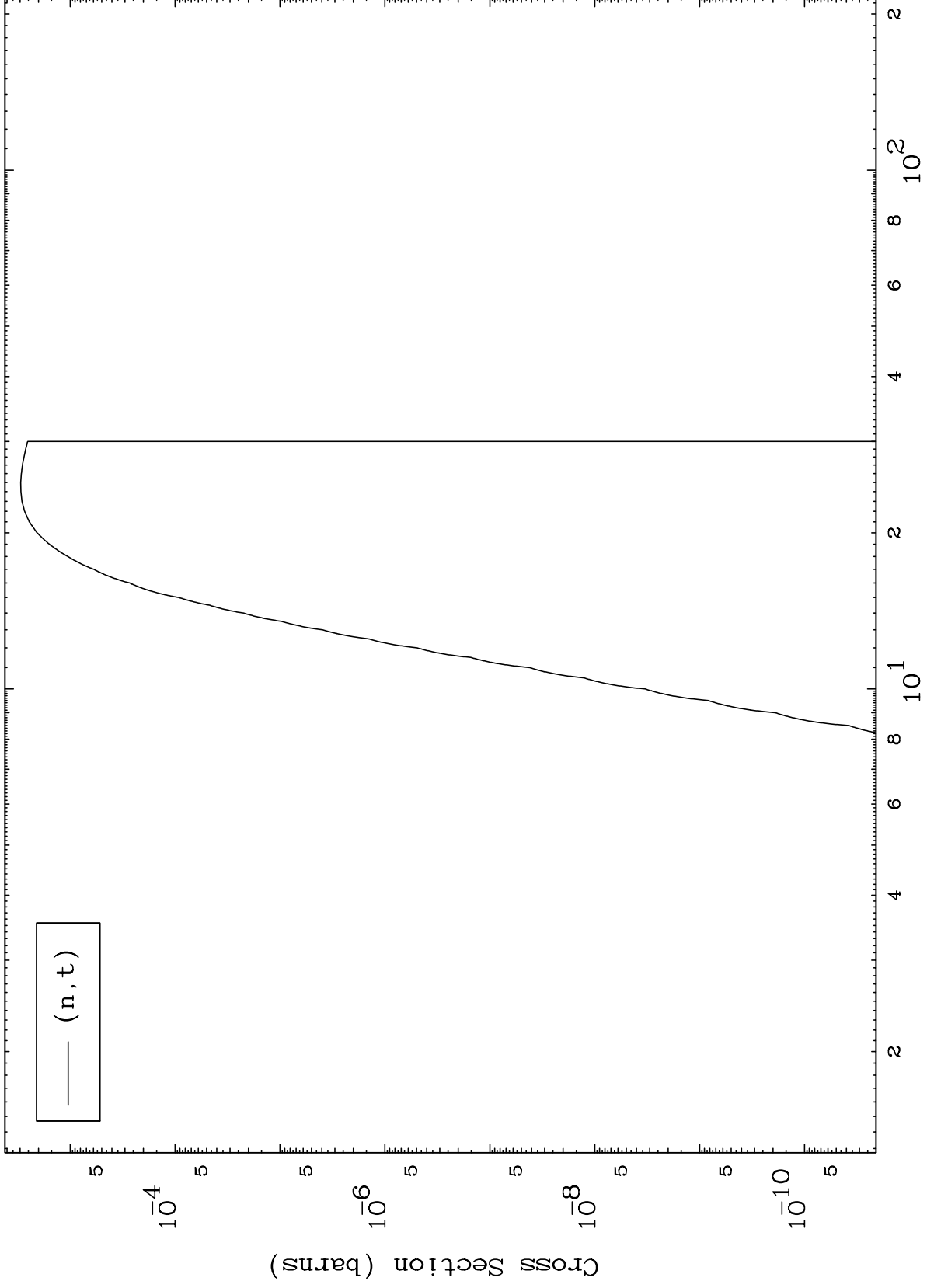
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n,t) Levels
293 Kelvin Cross Sections

97-Bk-248



15

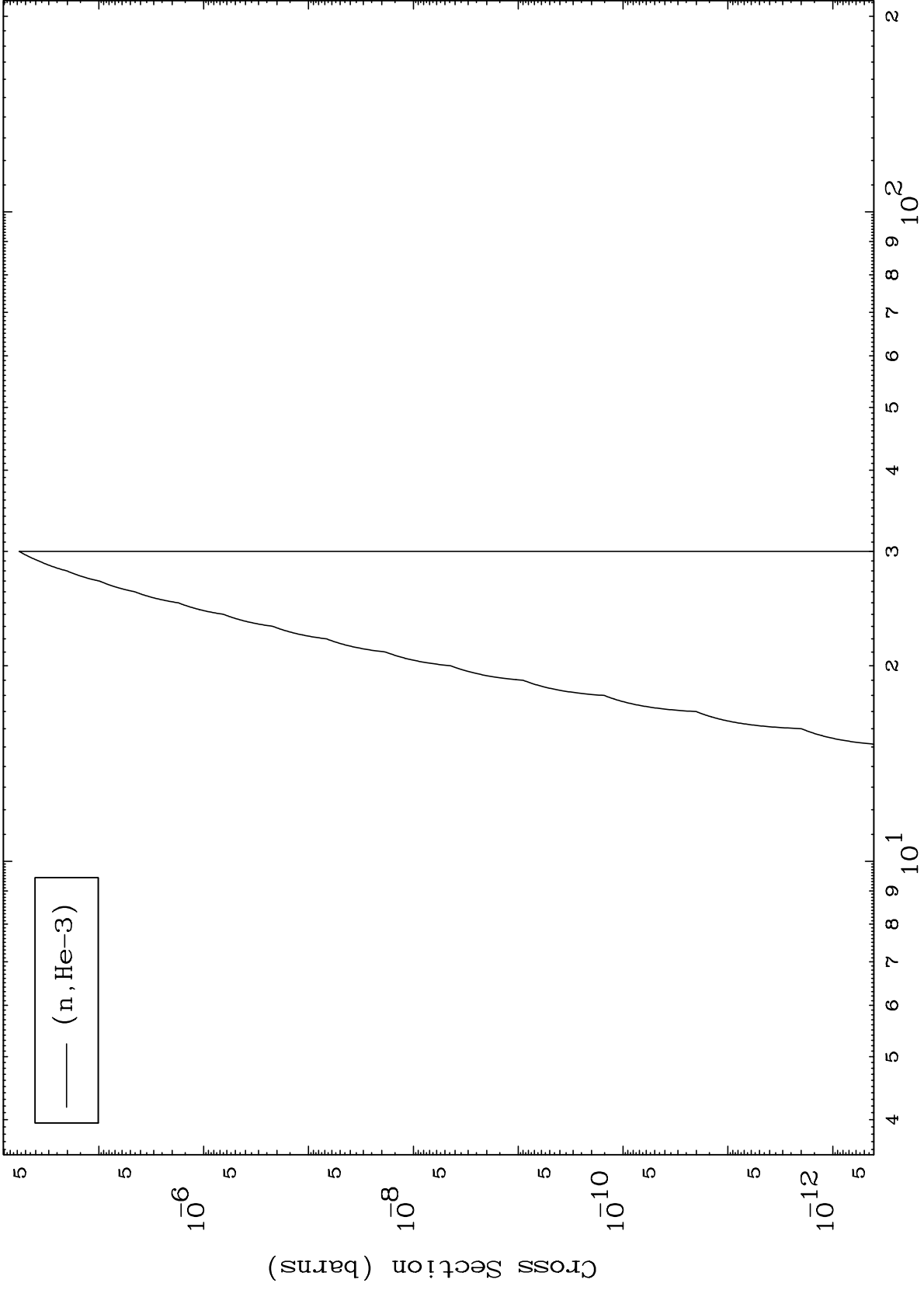
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-248



16

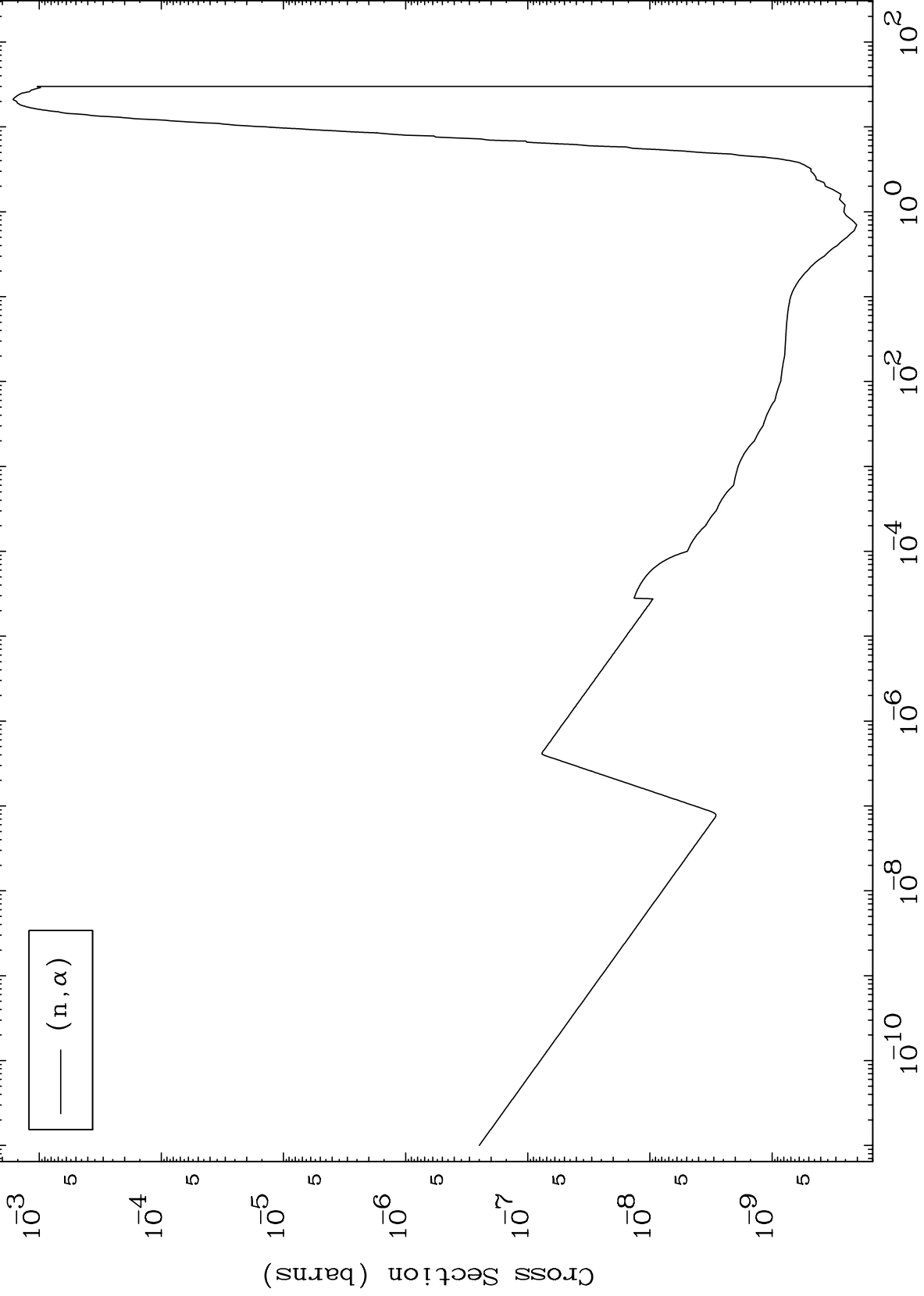
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n, α) Levels
293 Kelvin Cross Sections

97-Bk-248



17

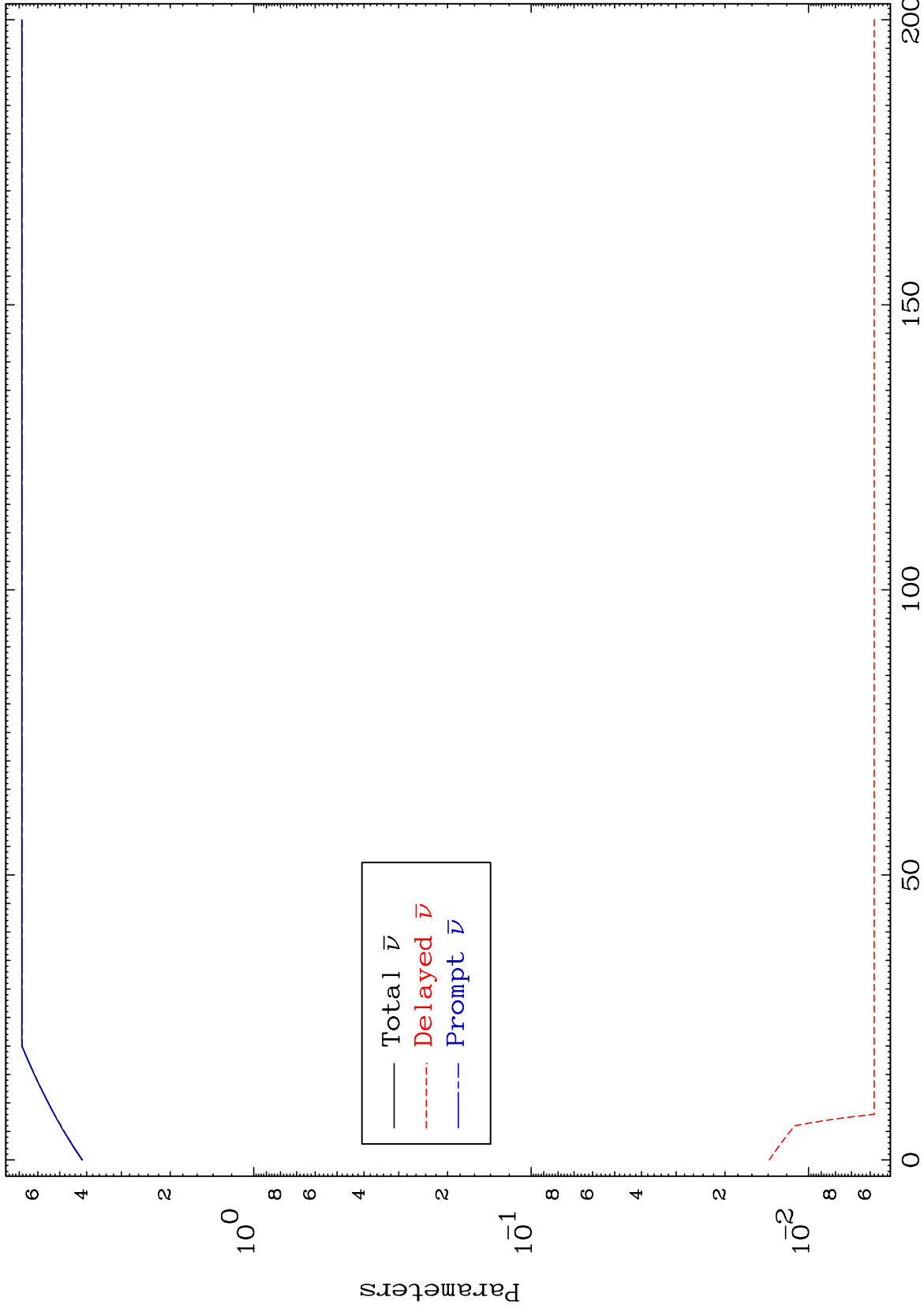
Incident Energy (MeV)

97-Bk-248

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Energy Release
Parameters

97-Bk-248



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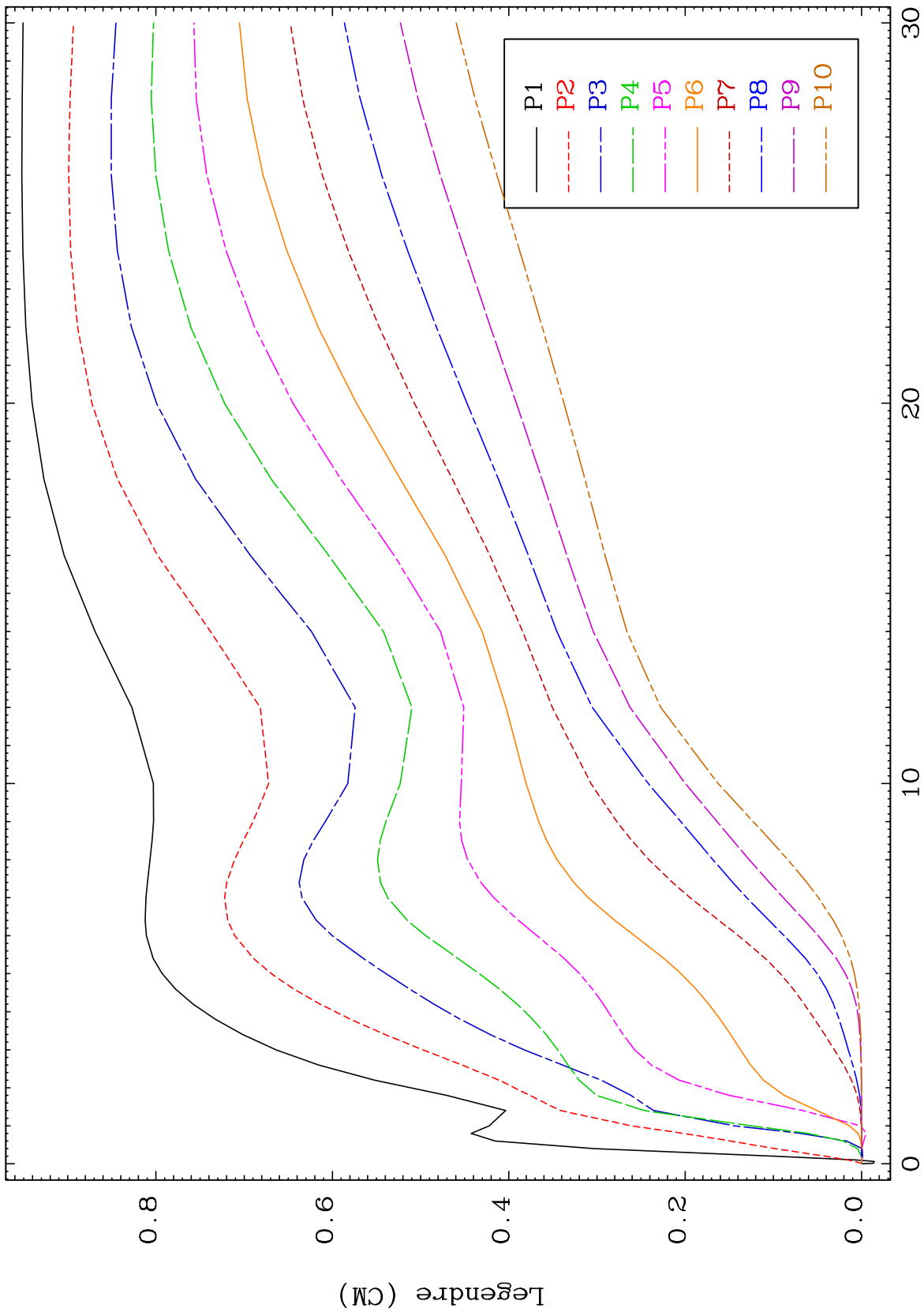
Incident Energy (MeV)

97-Bk-248

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Elastic Legendre Coefficients

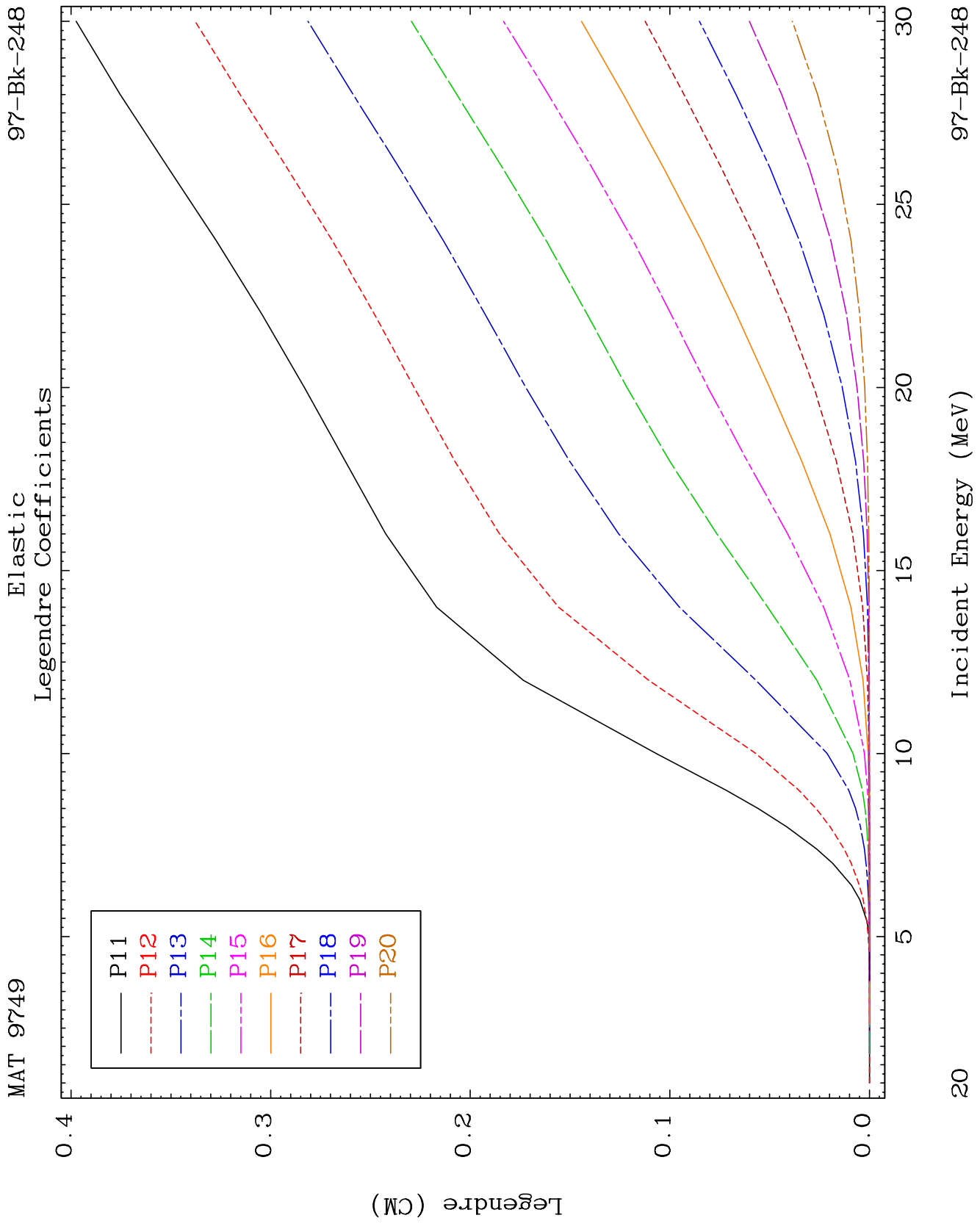
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Incident Energy (MeV)

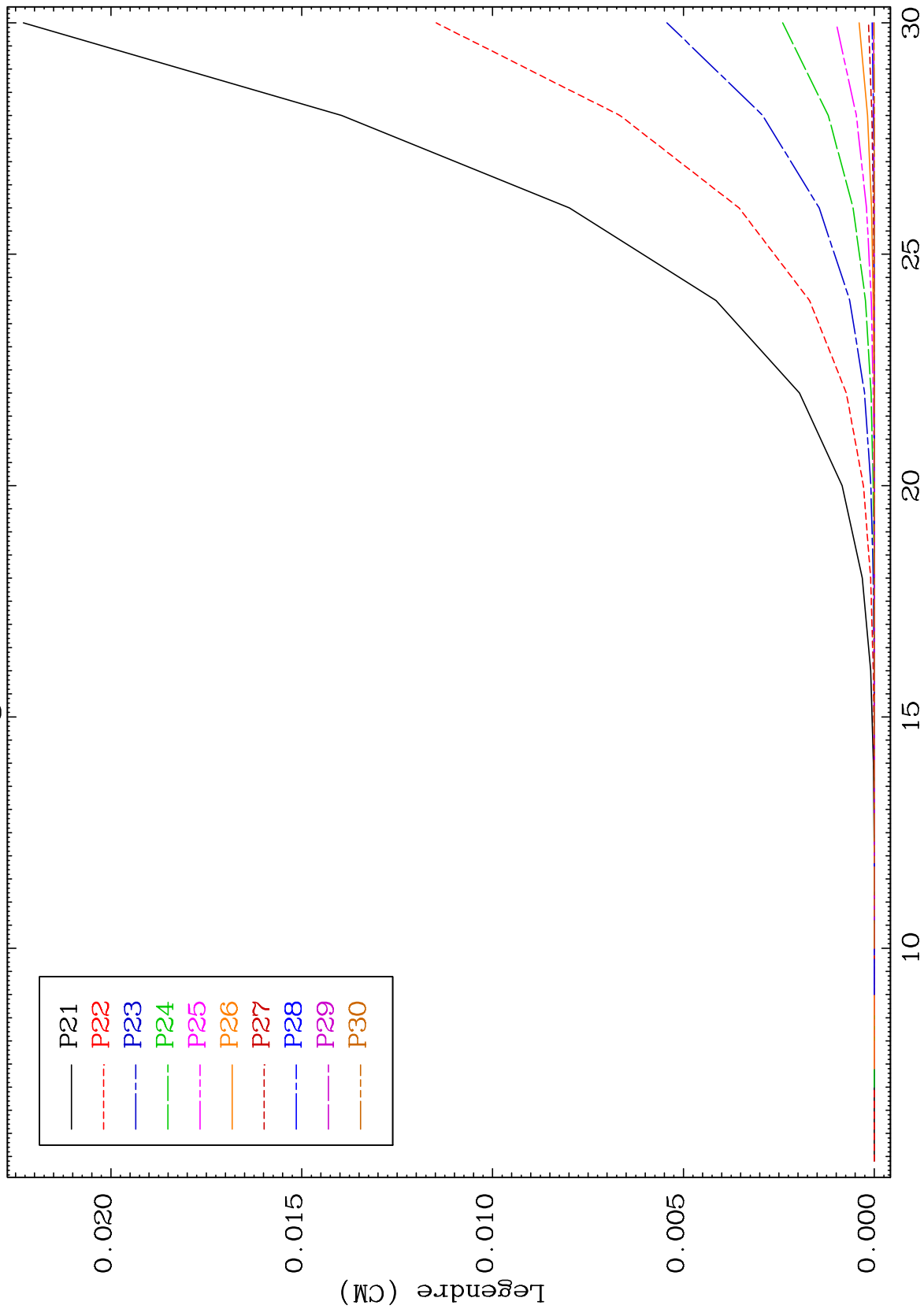
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MAT 9749

Elastic Legendre Coefficients

97-Bk-248



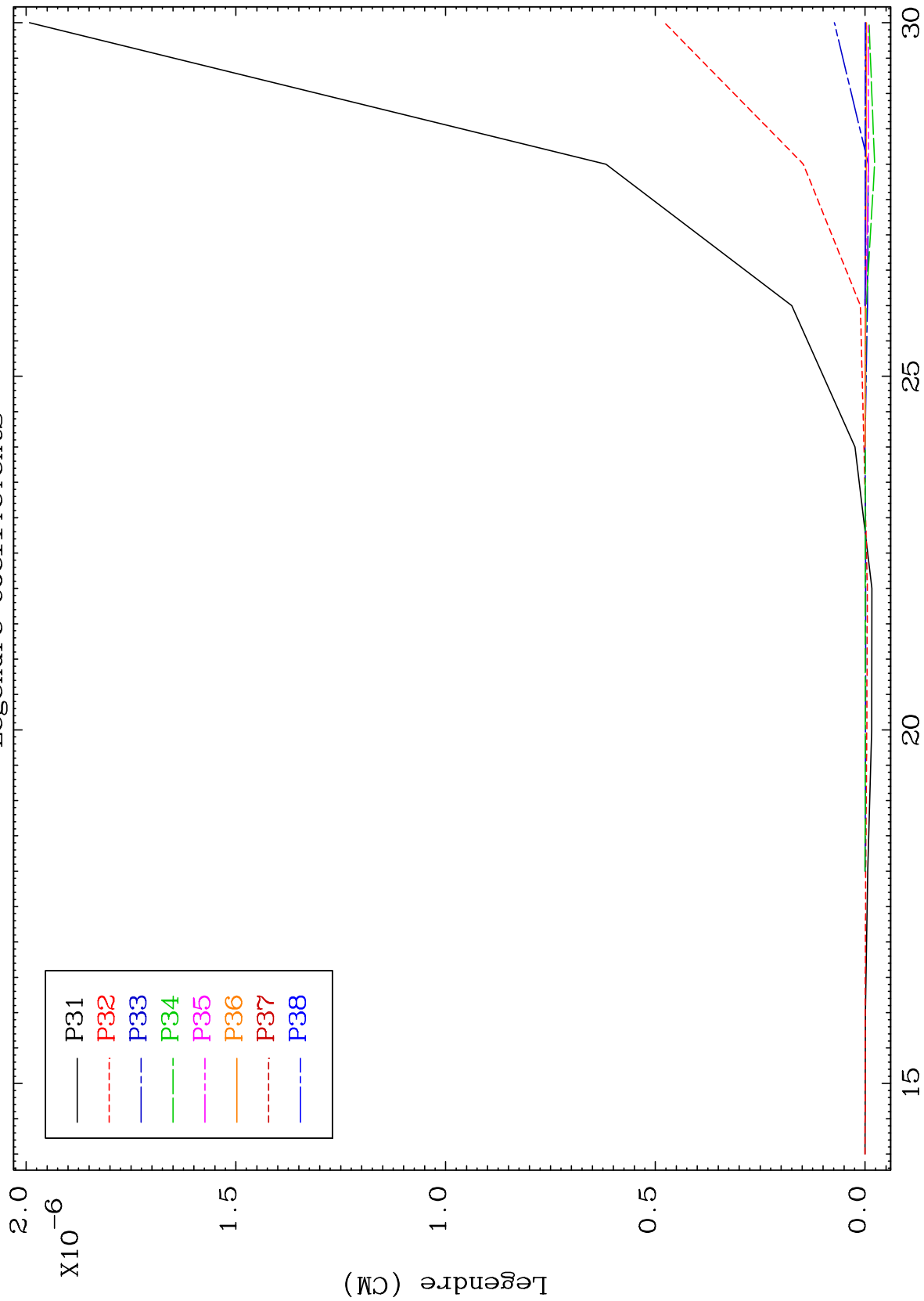
97-Bk-248

Incident Energy (MeV)

MAT 9749

Elastic
Legendre Coefficients

97-Bk-248



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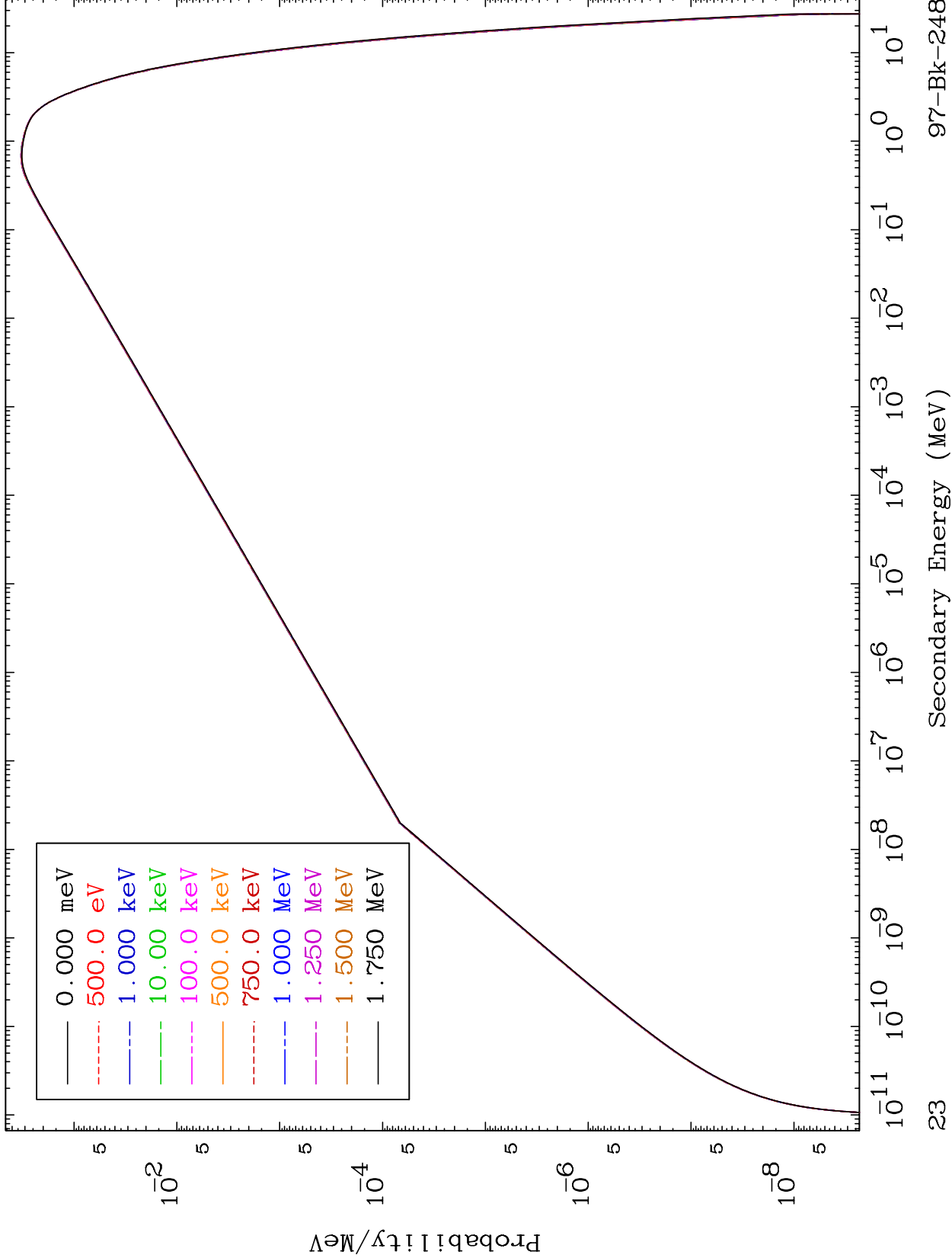
Incident Energy (MeV)

97-Bk-248

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Fission Energy Distributions

97-Bk-248



- 0.000 meV
- 500.0 eV
- 1.000 keV
- 10.00 keV
- 100.0 keV
- 500.0 keV
- 750.0 keV
- 1.000 MeV
- 1.250 MeV
- 1.500 MeV
- 1.750 MeV

97-Bk-248

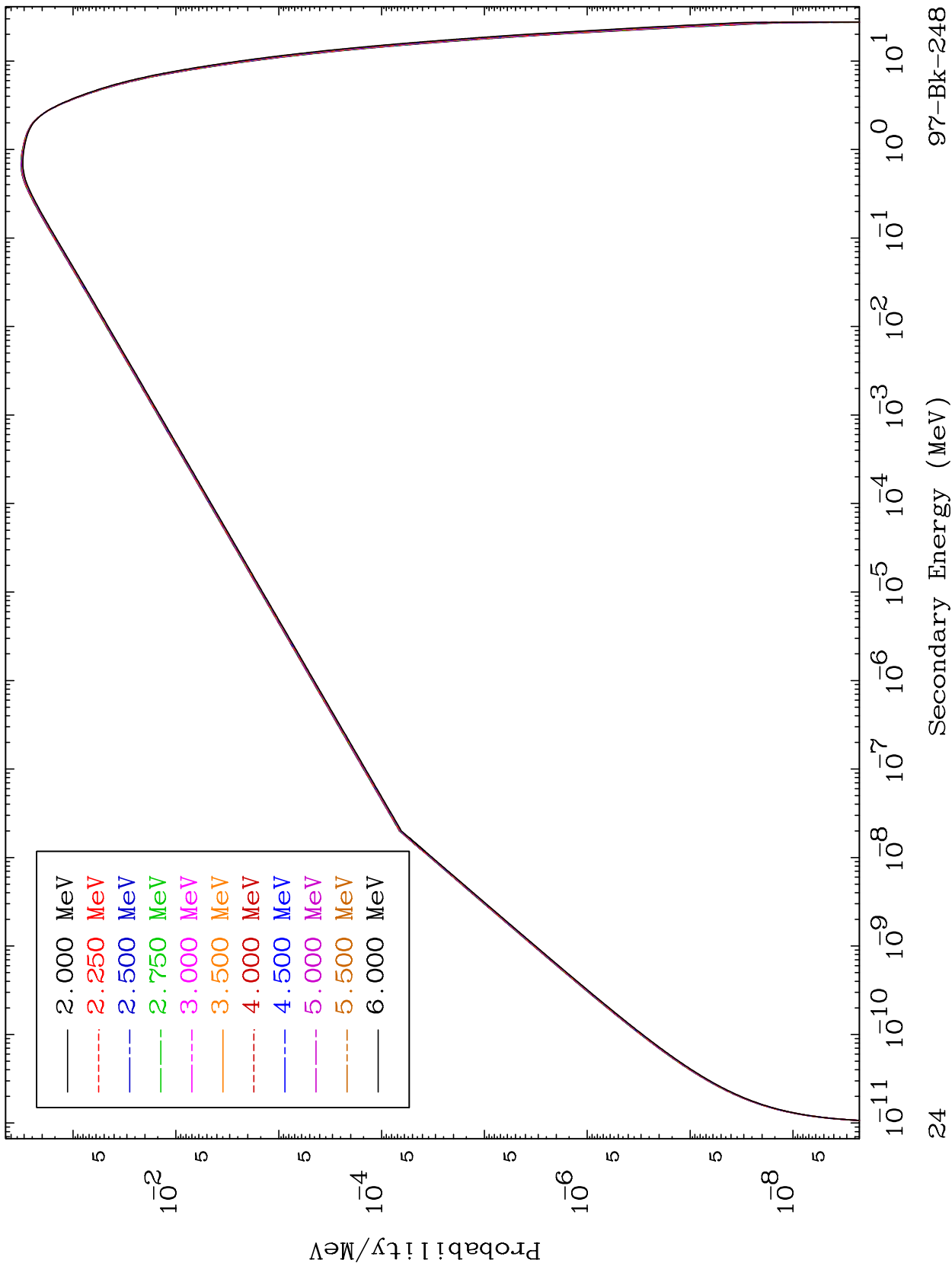
Secondary Energy (MeV)

23

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Fission Energy Distributions

97-Bk-248



97-Bk-248

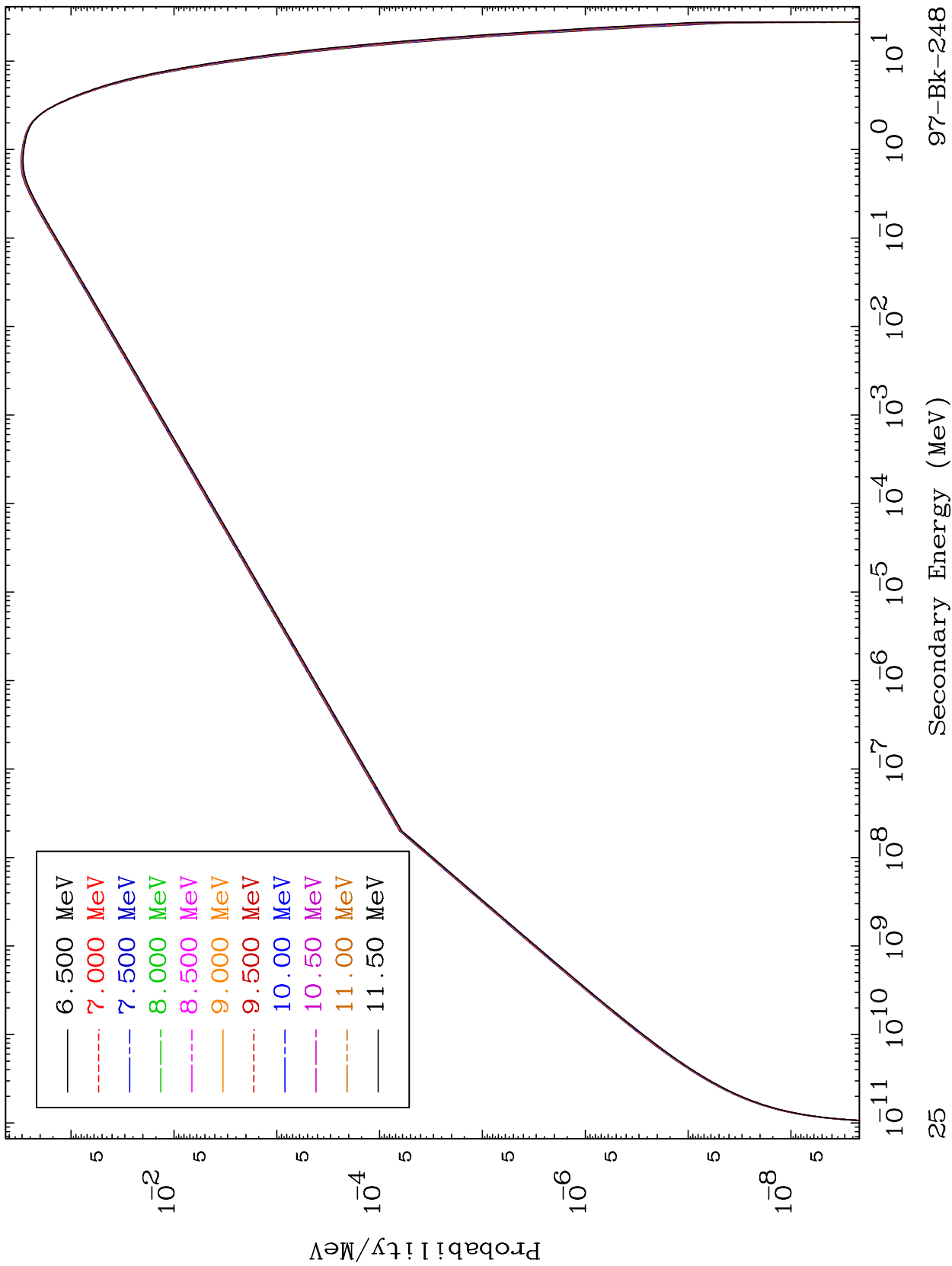
Secondary Energy (MeV)

24

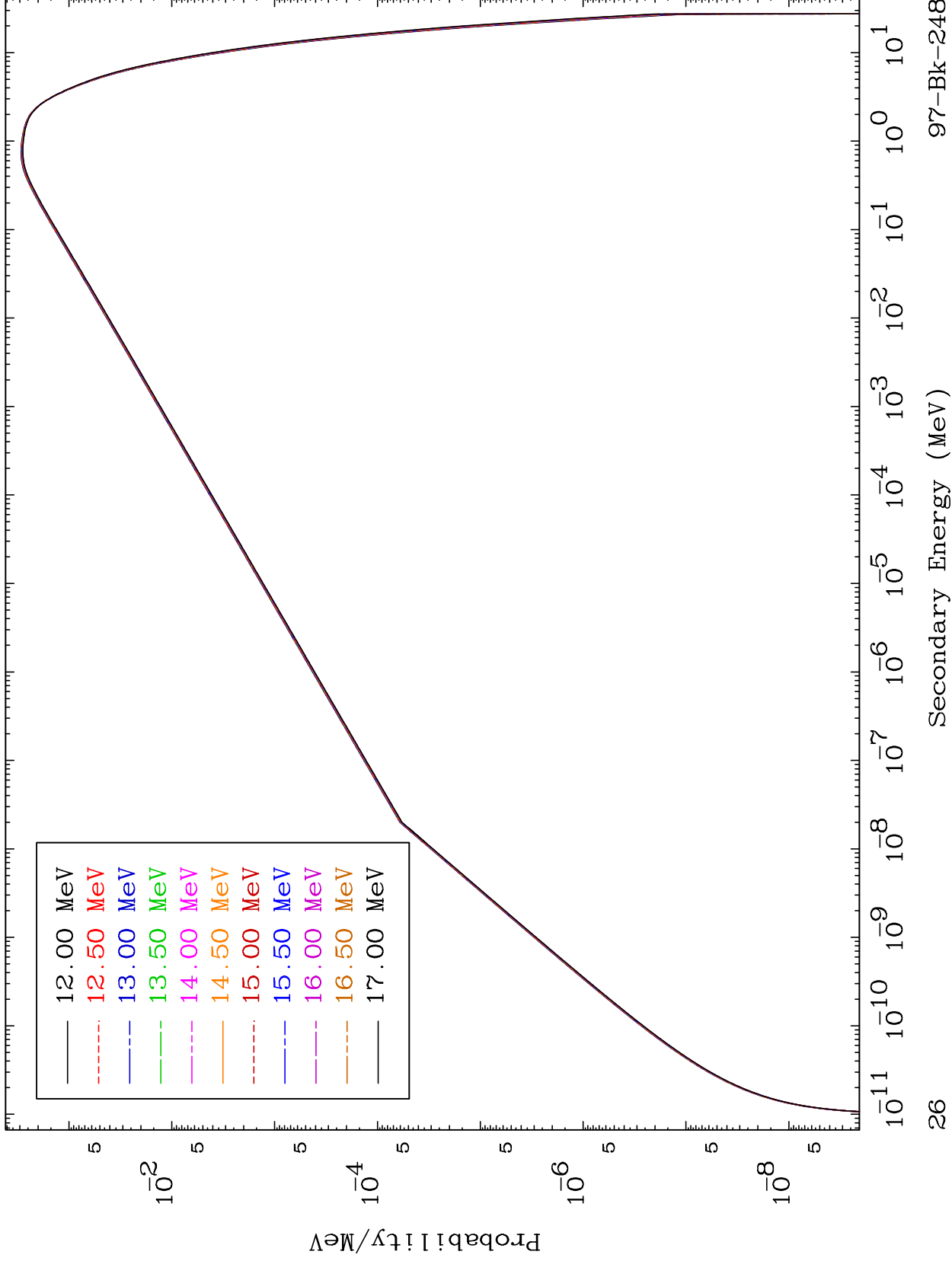
MAT 9749

Fission Energy Distributions

97-Bk-248



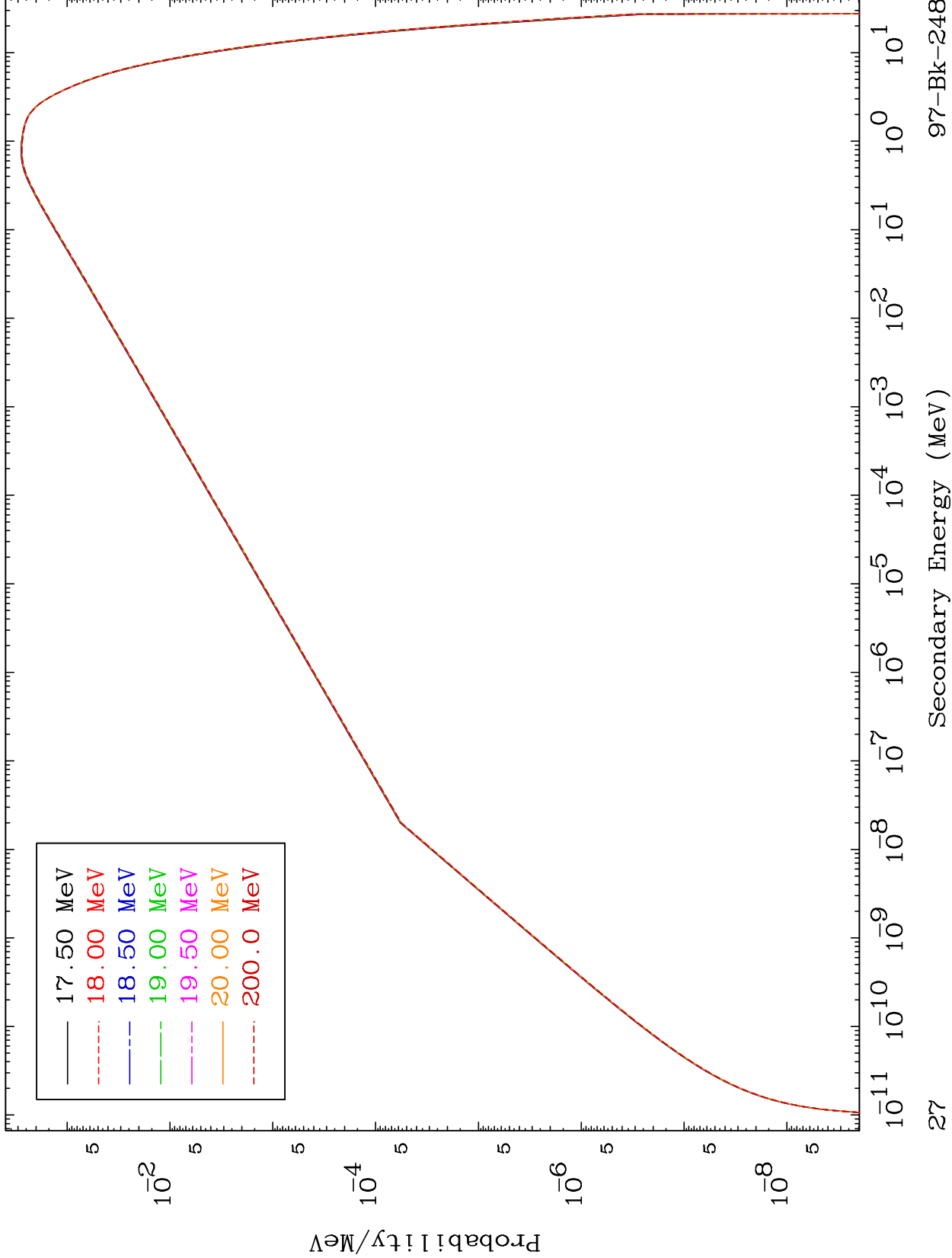
Fission Energy Distributions



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Fission Energy Distributions

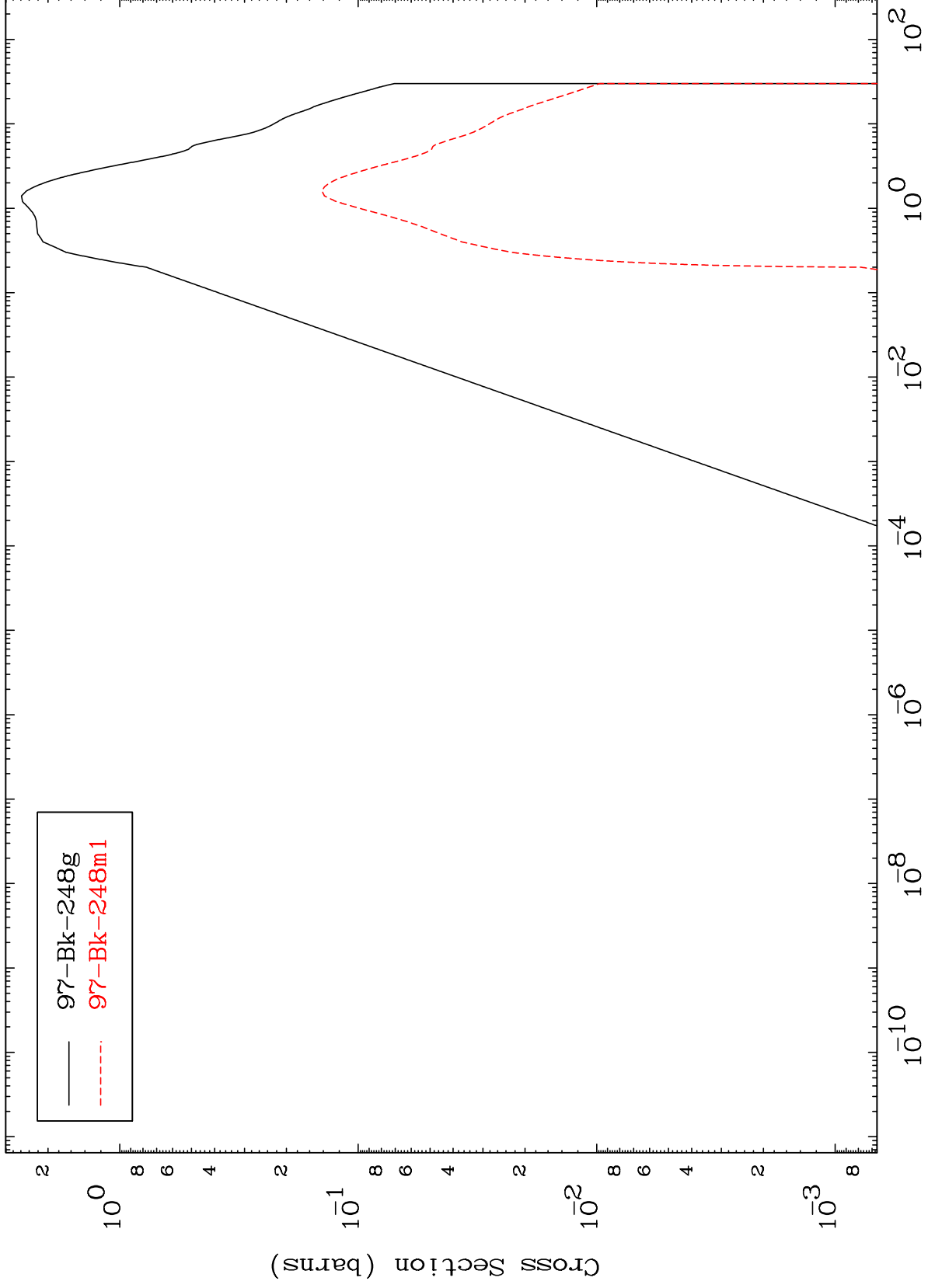
97-Bk-248



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Inelastic
Radionuclide Production Cross Section

97-Bk-248



— 97-Bk-248g
- - - 97-Bk-248m1

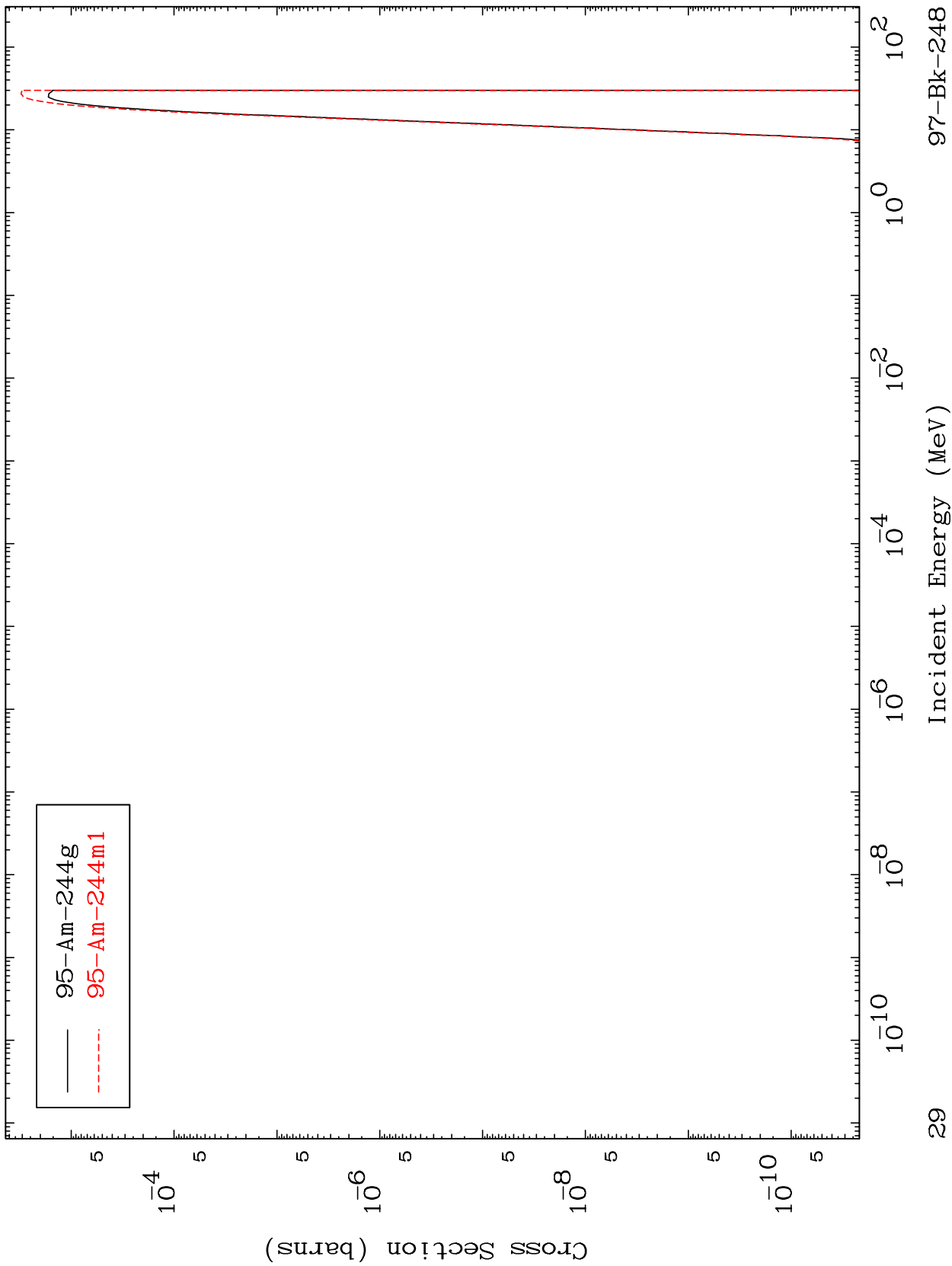
97-Bk-248

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(n,n') α

97-Bk-248

Radionuclide Production Cross Section



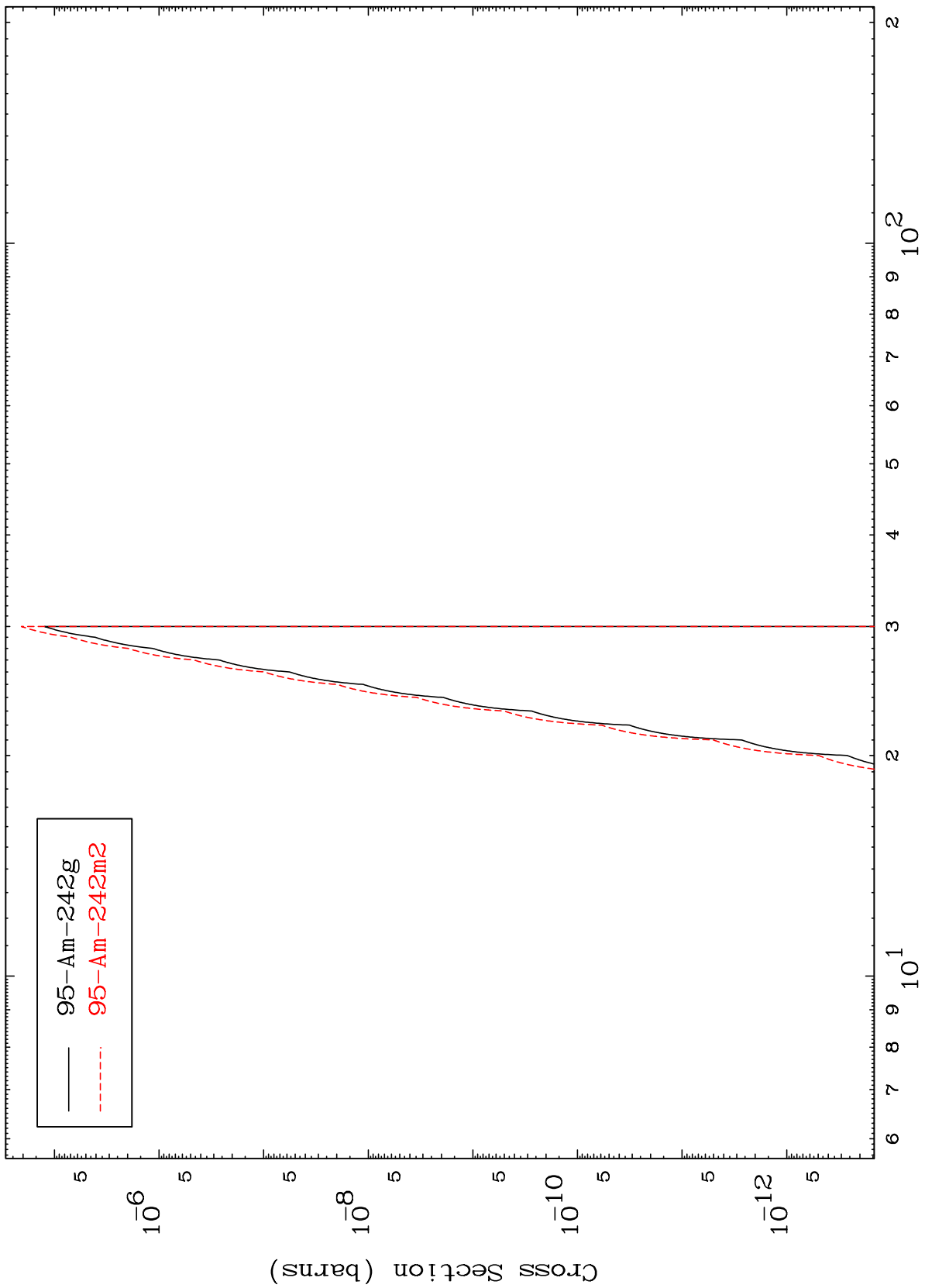
95-Am-244g
95-Am-244m1

MAT 9749

(n,3n) α

97-Bk-248

Radionuclide Production Cross Section



30

Incident Energy (MeV)

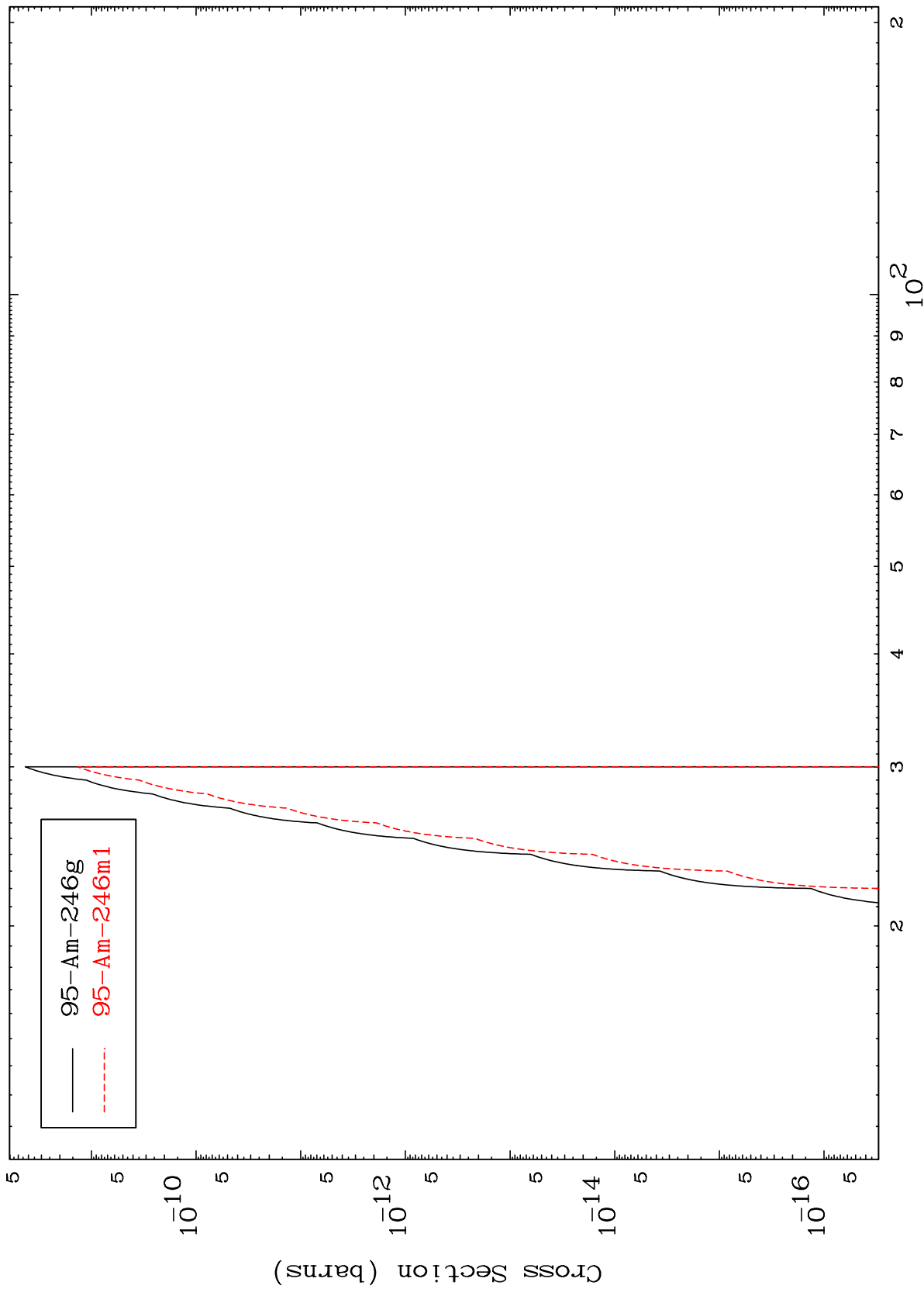
97-Bk-248

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(n,2n) p

97-Bk-248

Radionuclide Production Cross Section



Incident Energy (MeV)

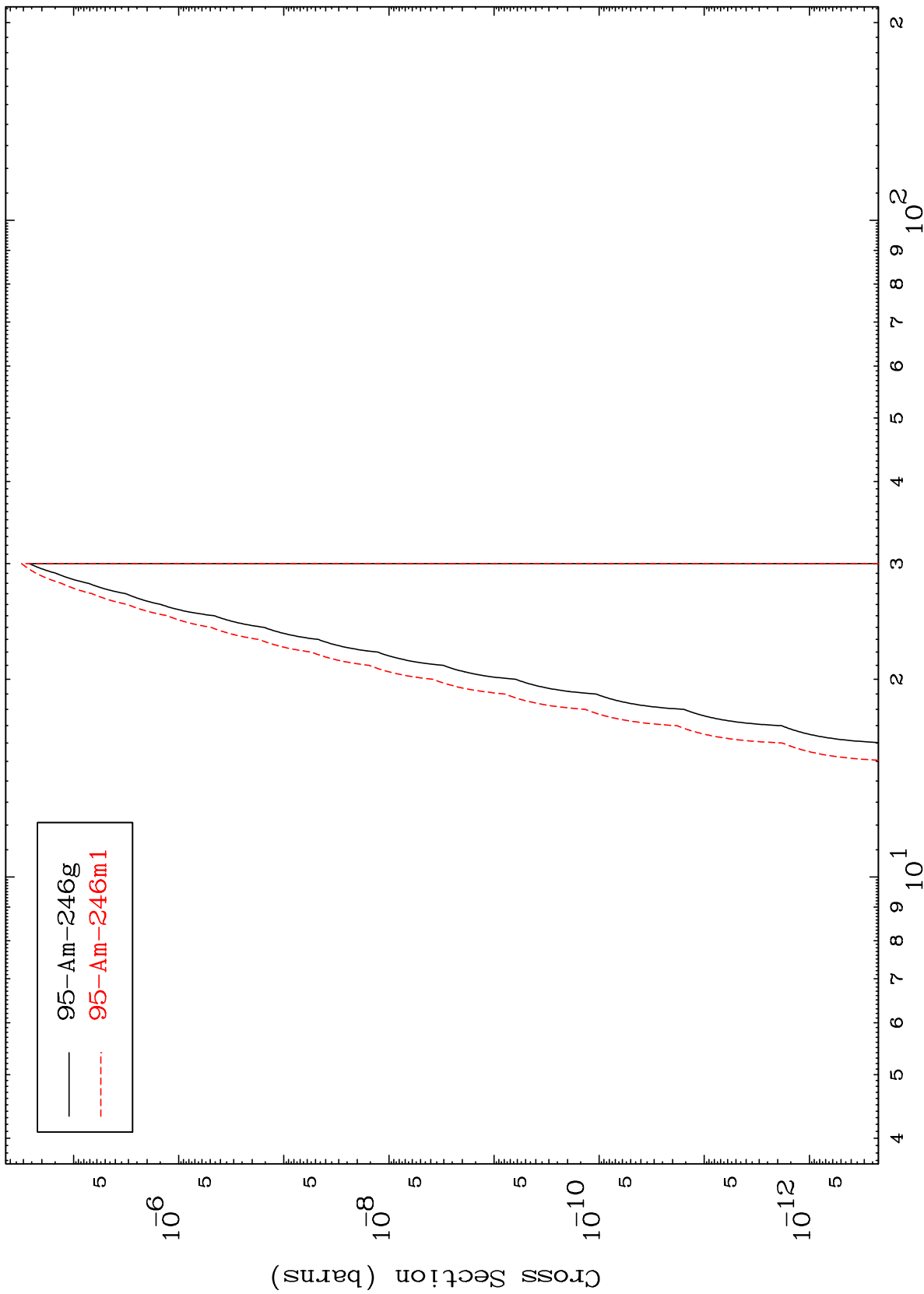
97-Bk-248

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97-Bk-248

Radionuclide Production Cross Section
(n,He-3)



Incident Energy (MeV)

97-Bk-248

32