

Program EVALPLOT  
(Version 2021-1)

by

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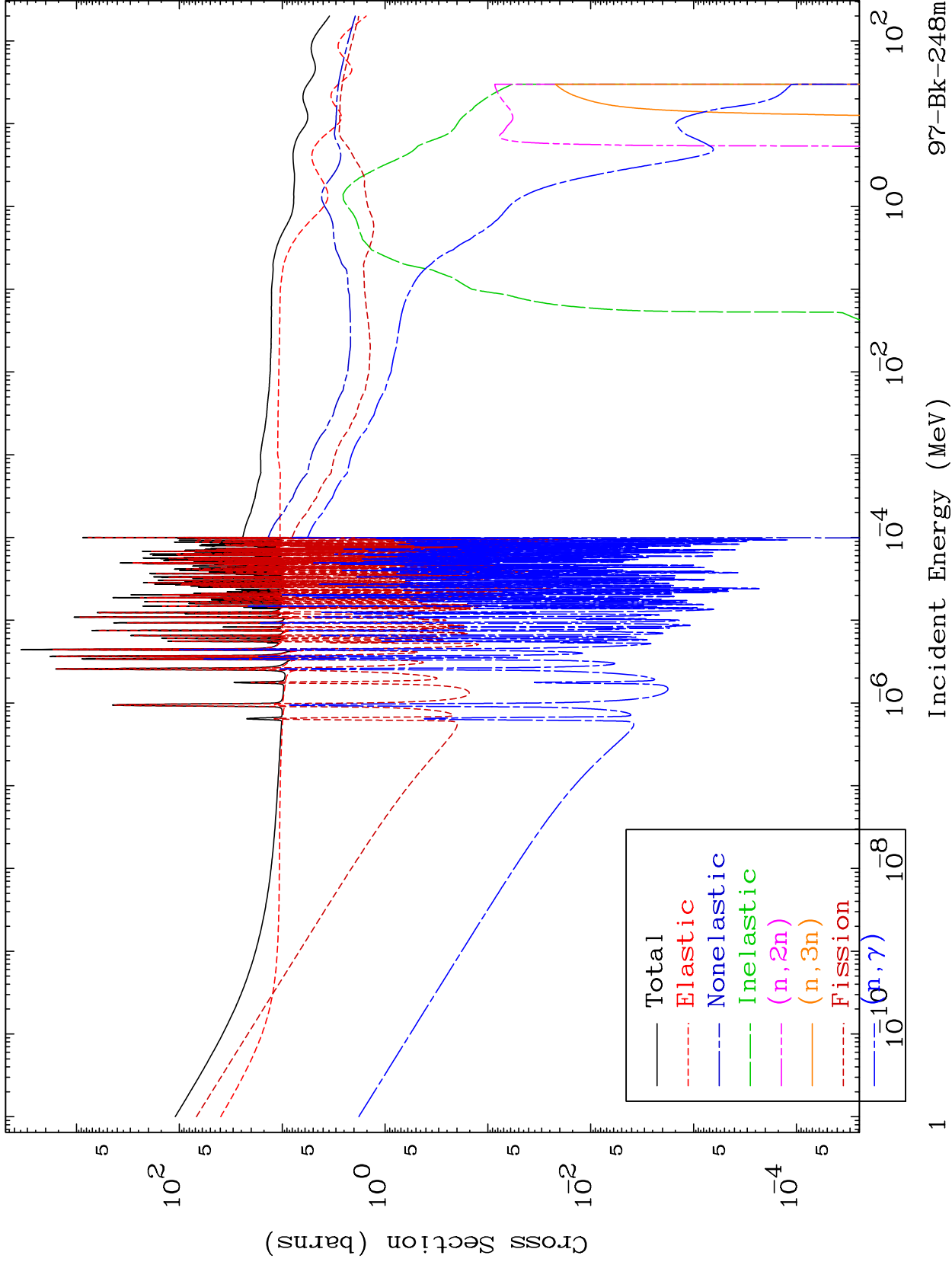
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9750

Neutron Major  
293 Kelvin Cross Sections

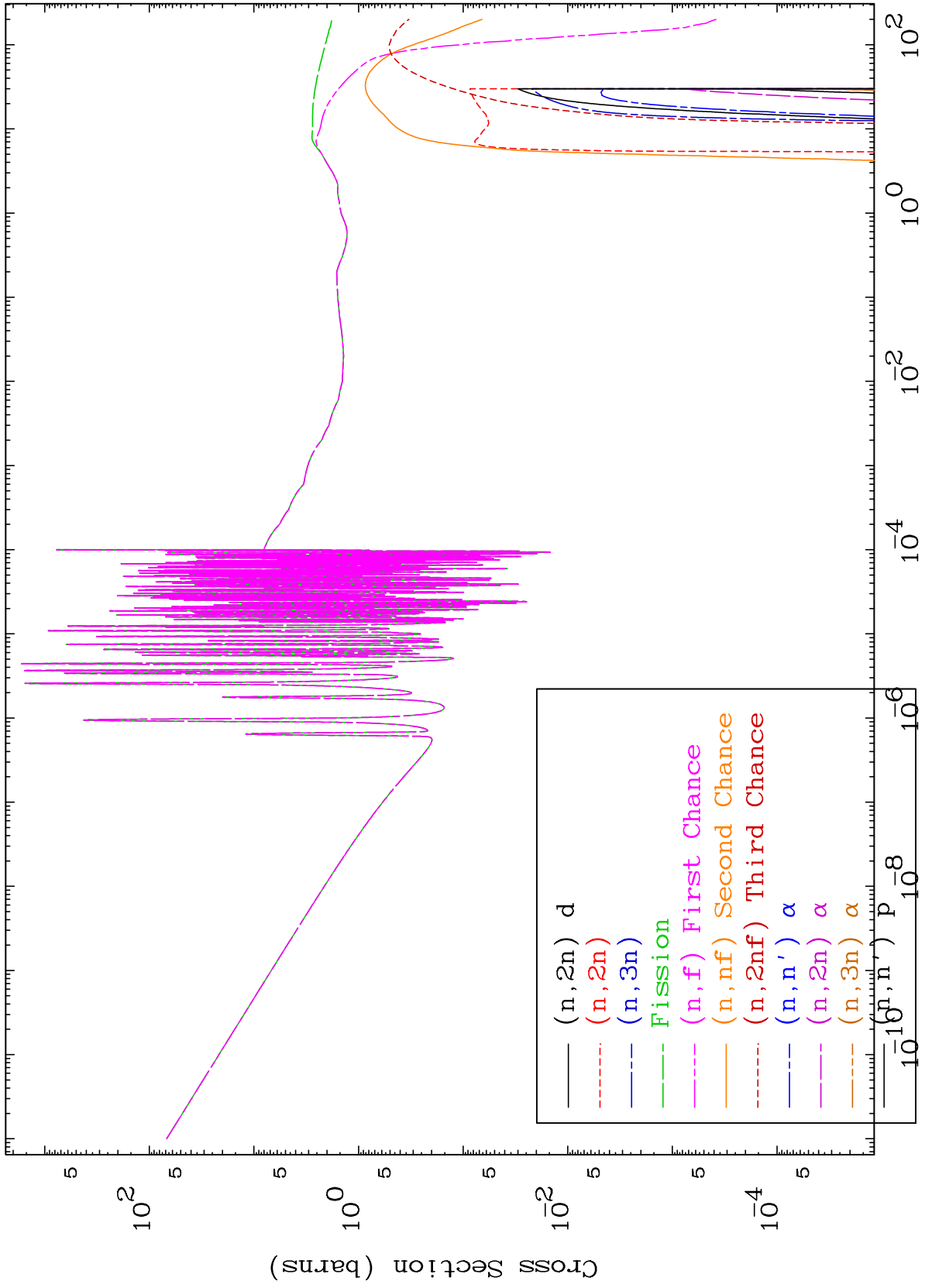
97-Bk-248m



MAT 9750

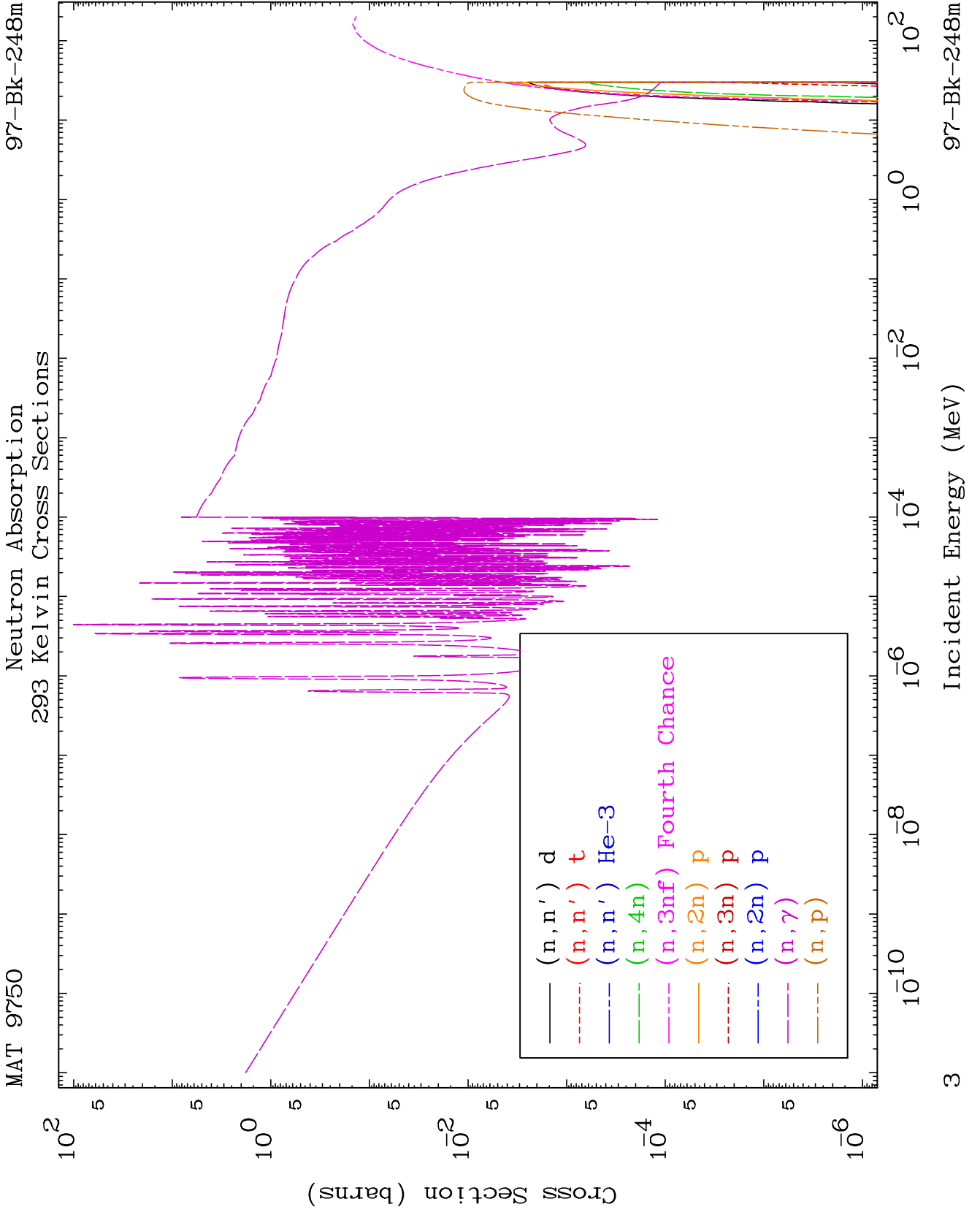
Neutron Absorption  
293 Kelvin Cross Sections

97-Bk-248m



97-Bk-248m

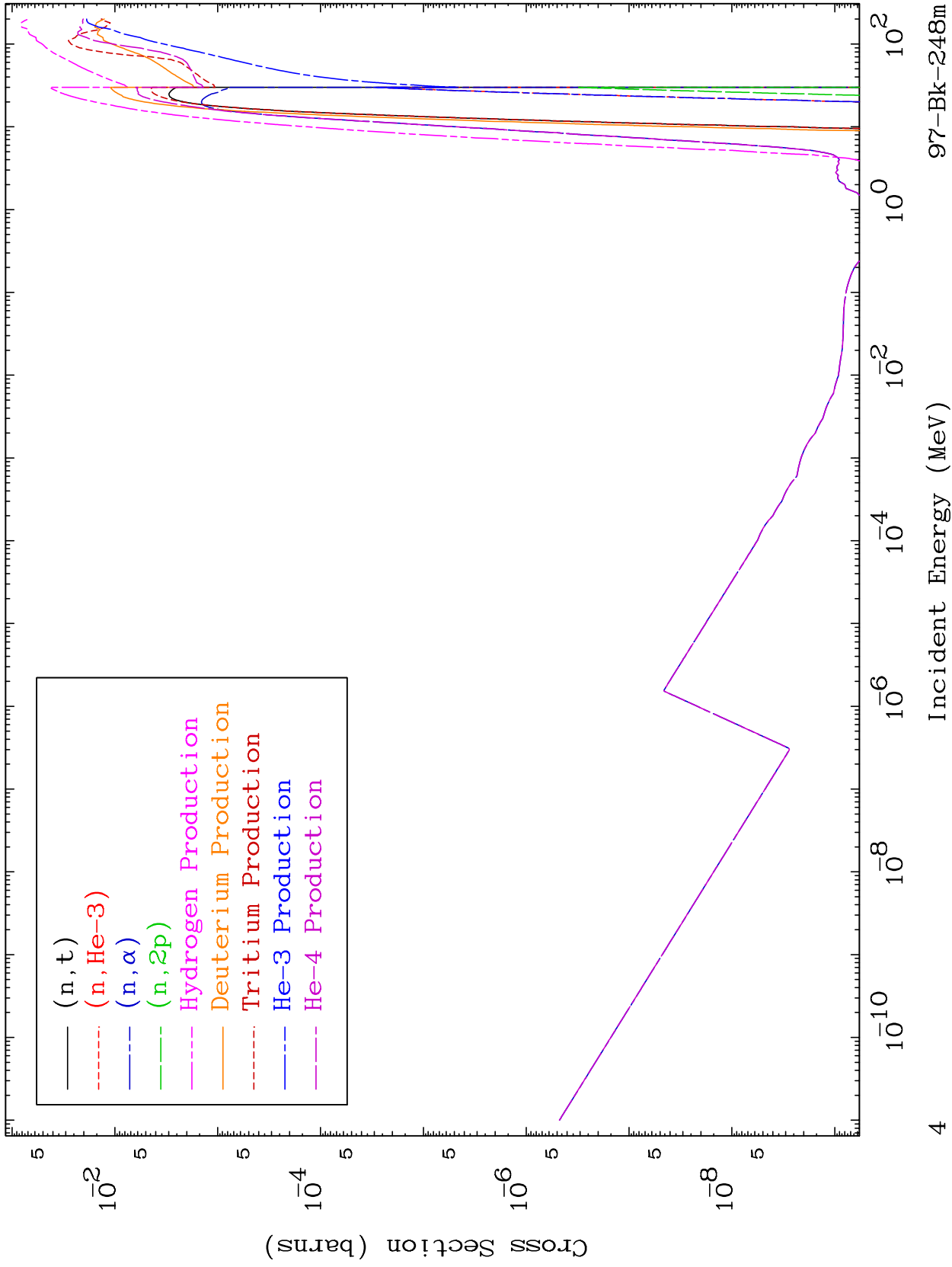
Incident Energy (MeV)



MAT 9750

Neutron Absorption  
293 Kelvin Cross Sections

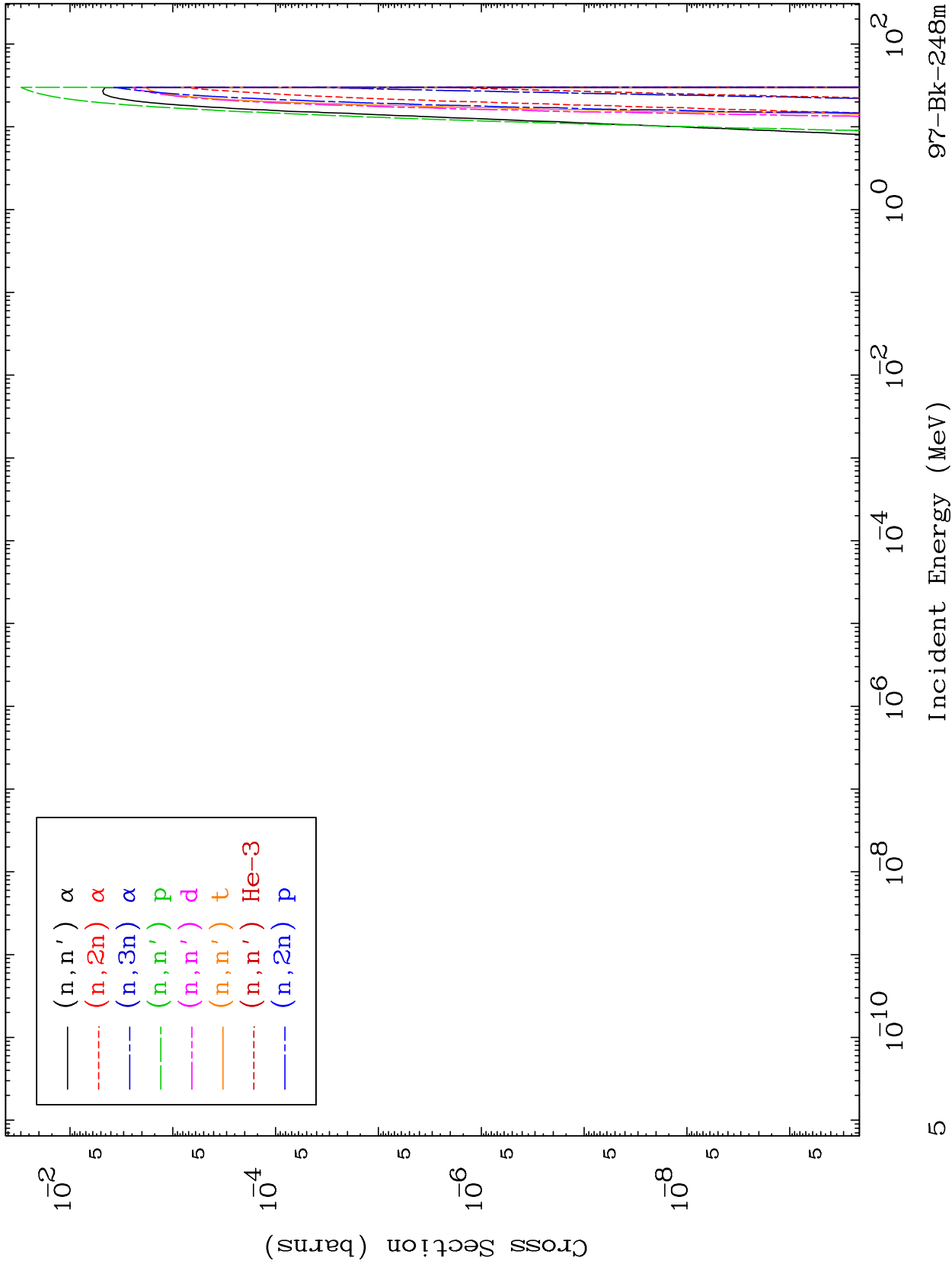
97-Bk-248m



MAT 9750

Charged Particle  
293 Kelvin Cross Sections

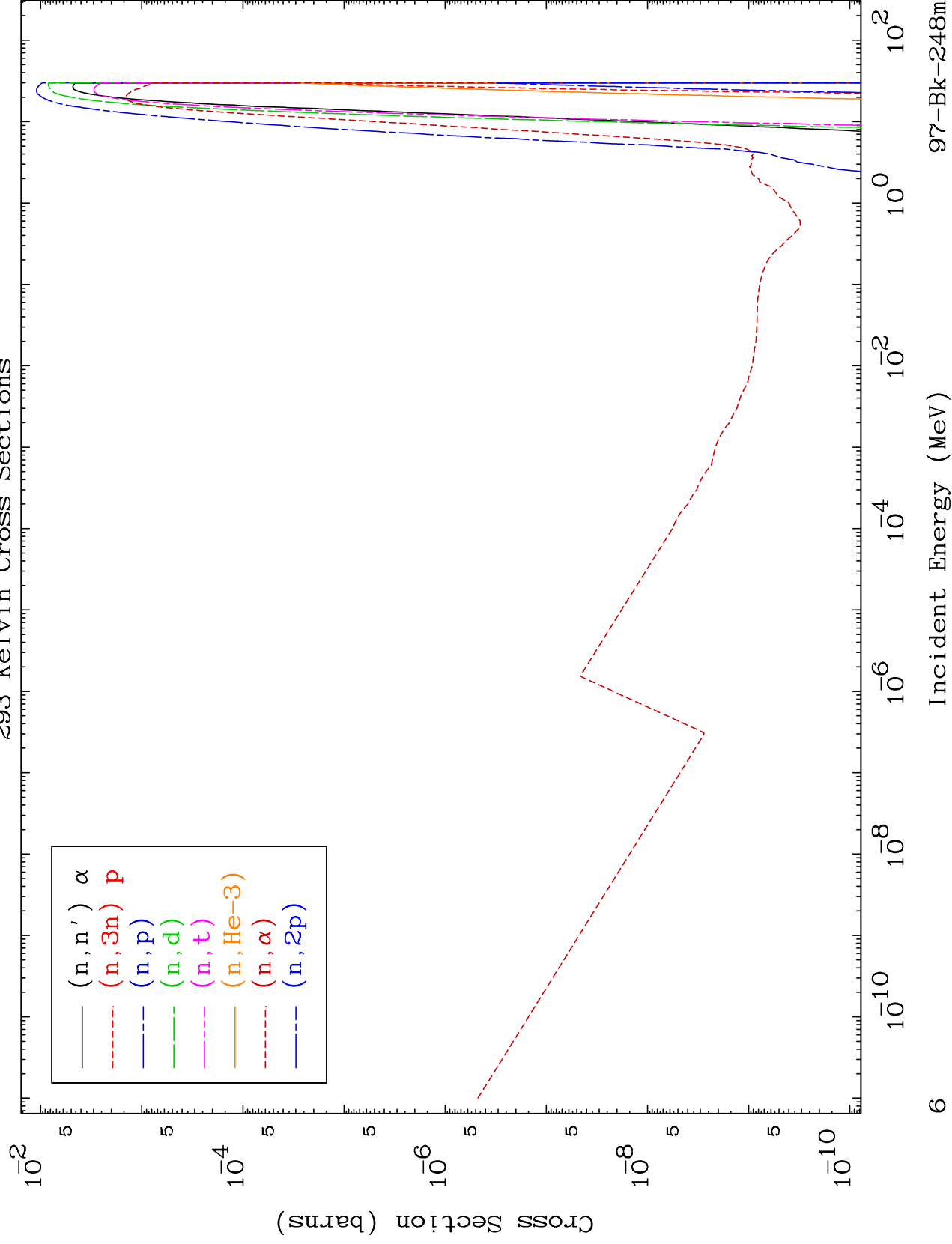
97-Bk-248m



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Charged Particle  
293 Kelvin Cross Sections

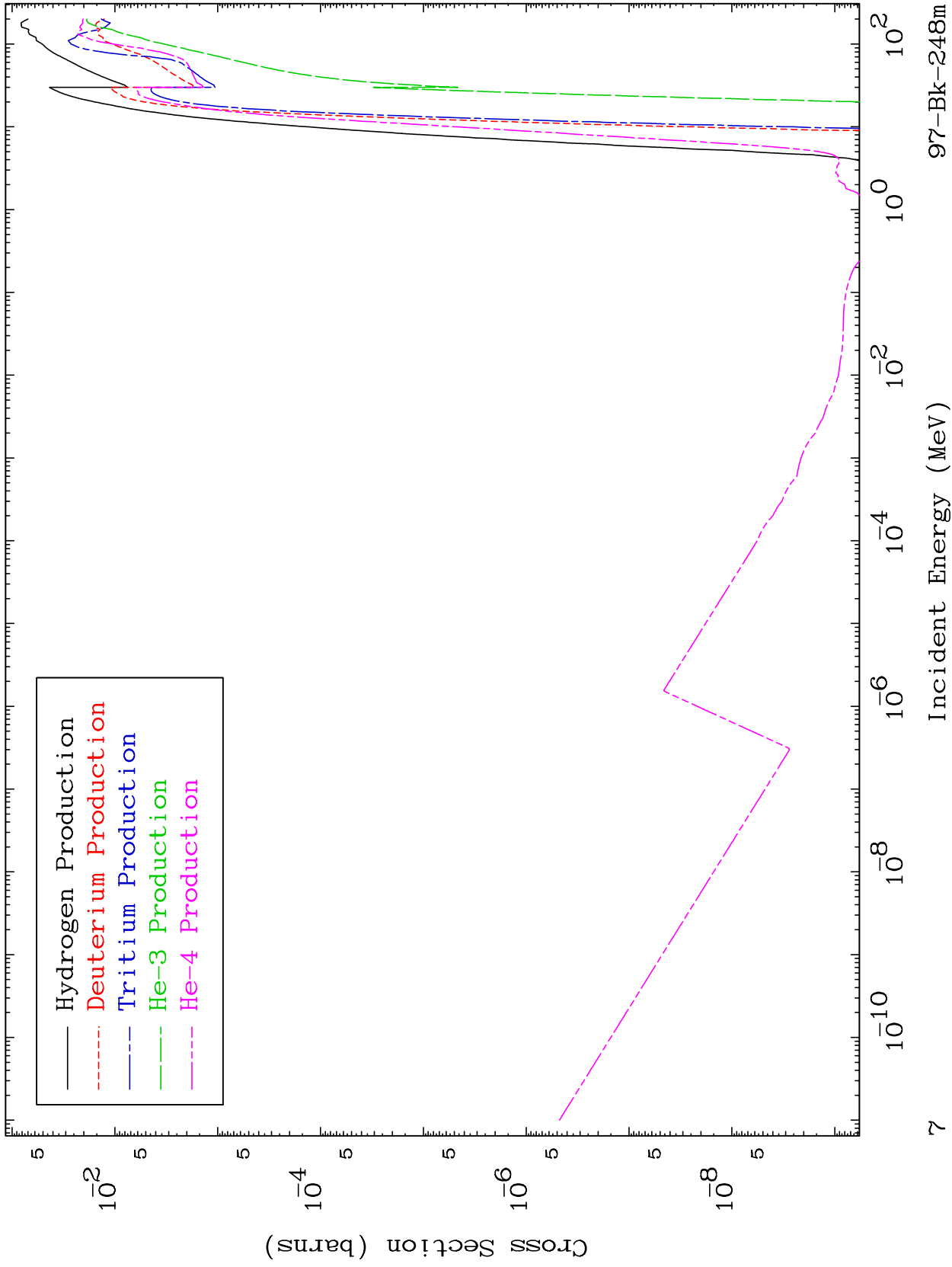
97-Bk-248m



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Particle Production  
293 Kelvin Cross Sections

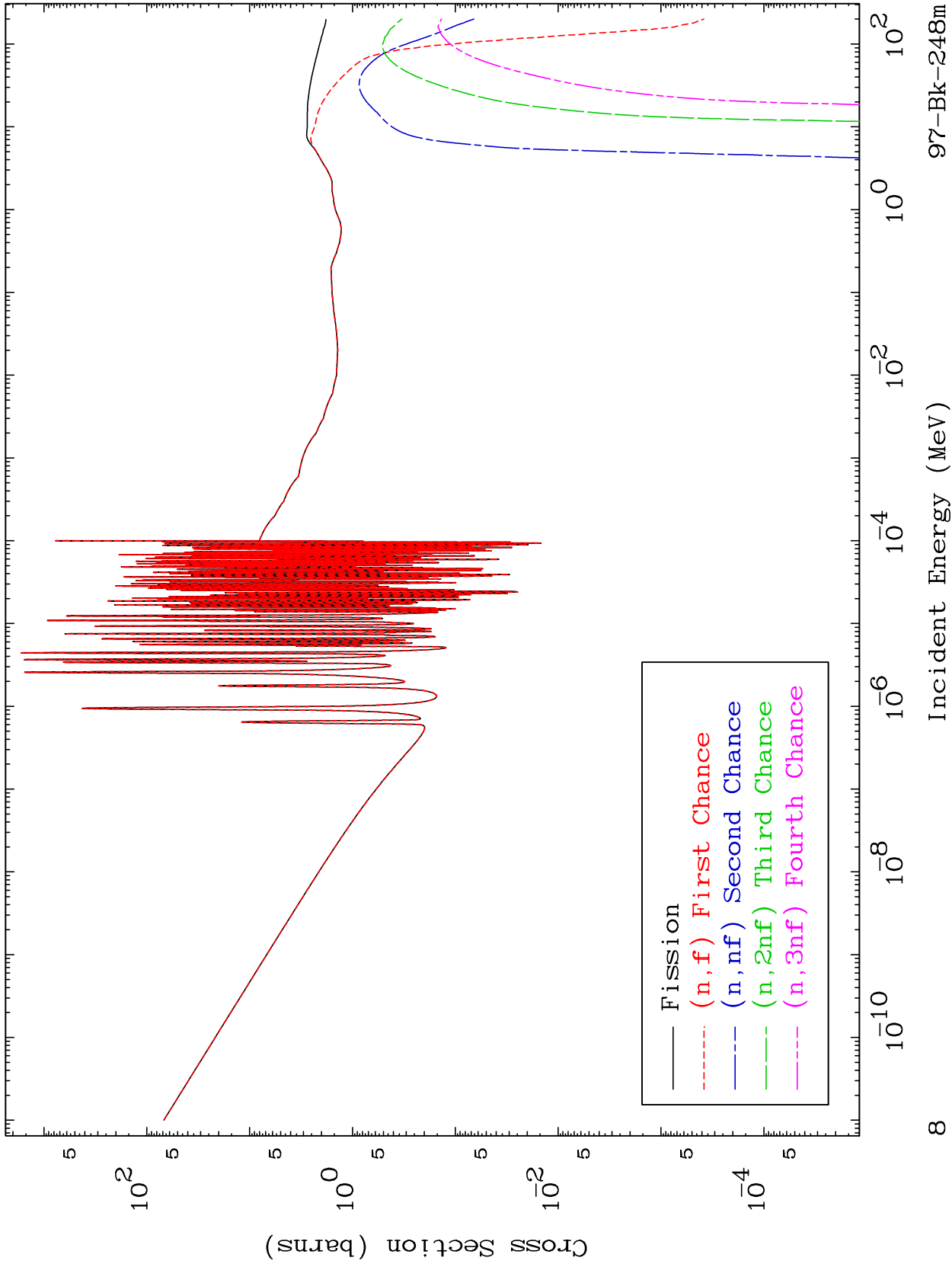
97-Bk-248m



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Fission  
293 Kelvin Cross Sections

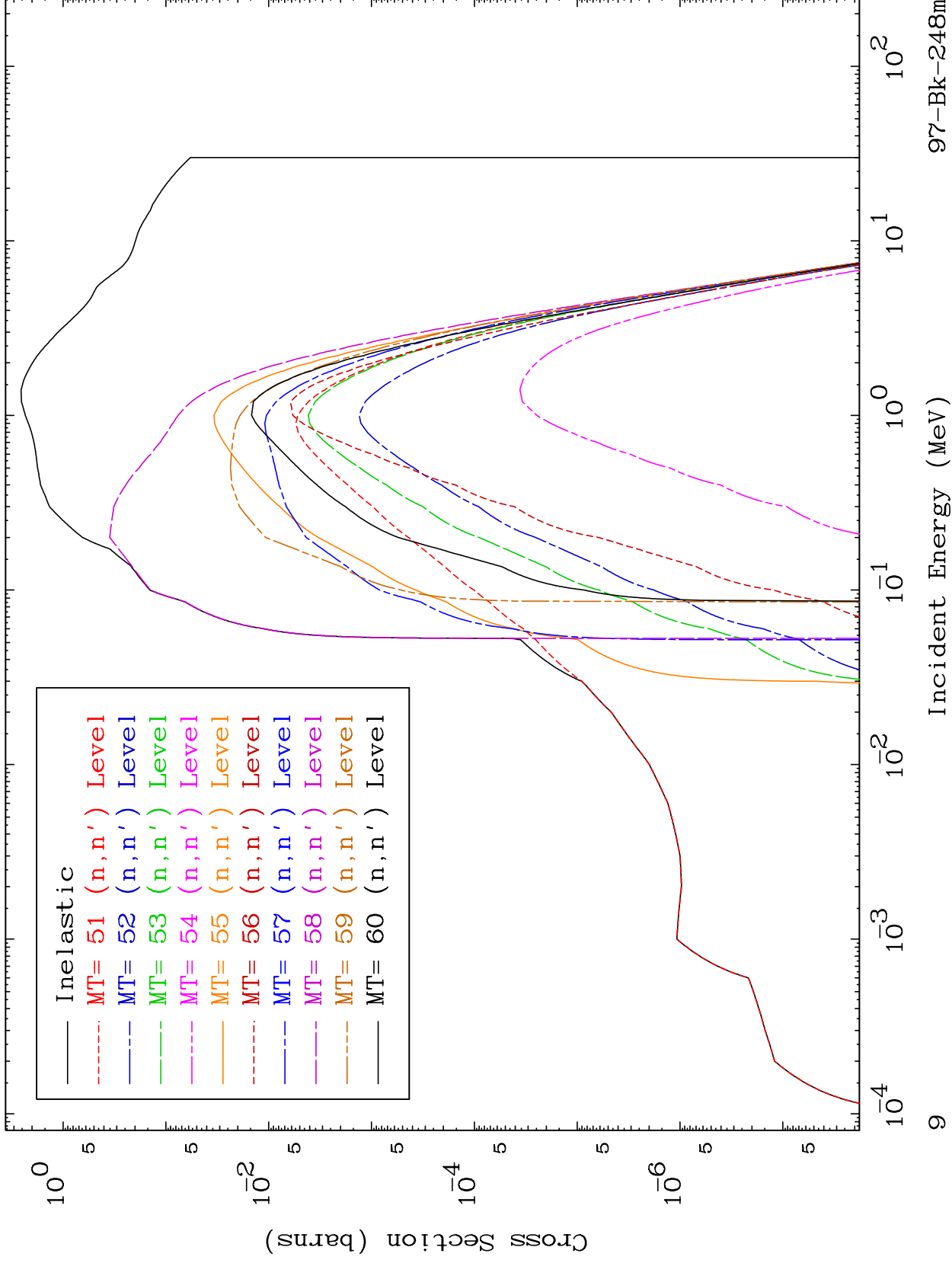
97-Bk-248m



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(n,n') Levels  
293 Kelvin Cross Sections

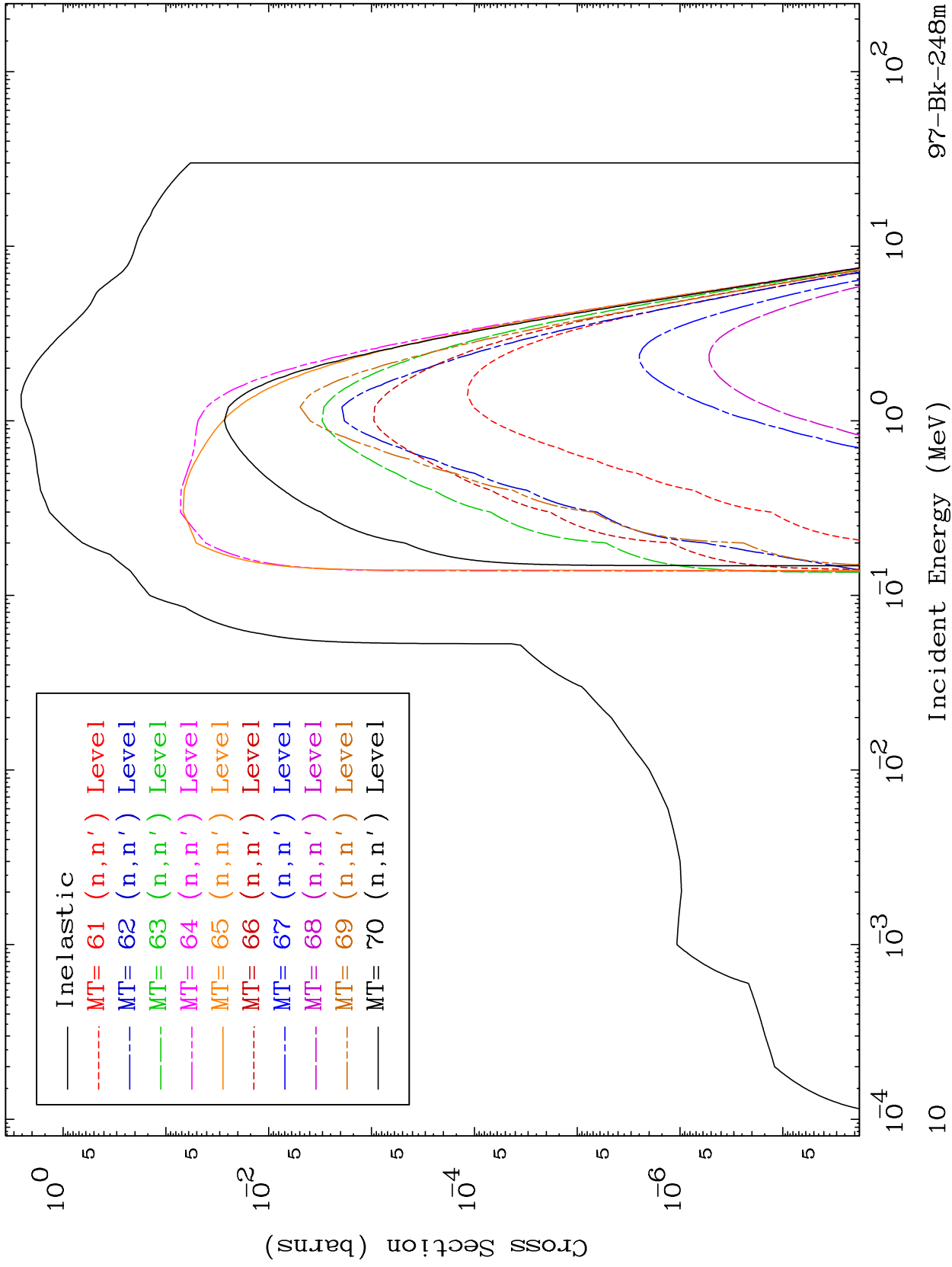
97-Bk-248m



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293 Kelvin Cross Sections  
(n,n') Levels

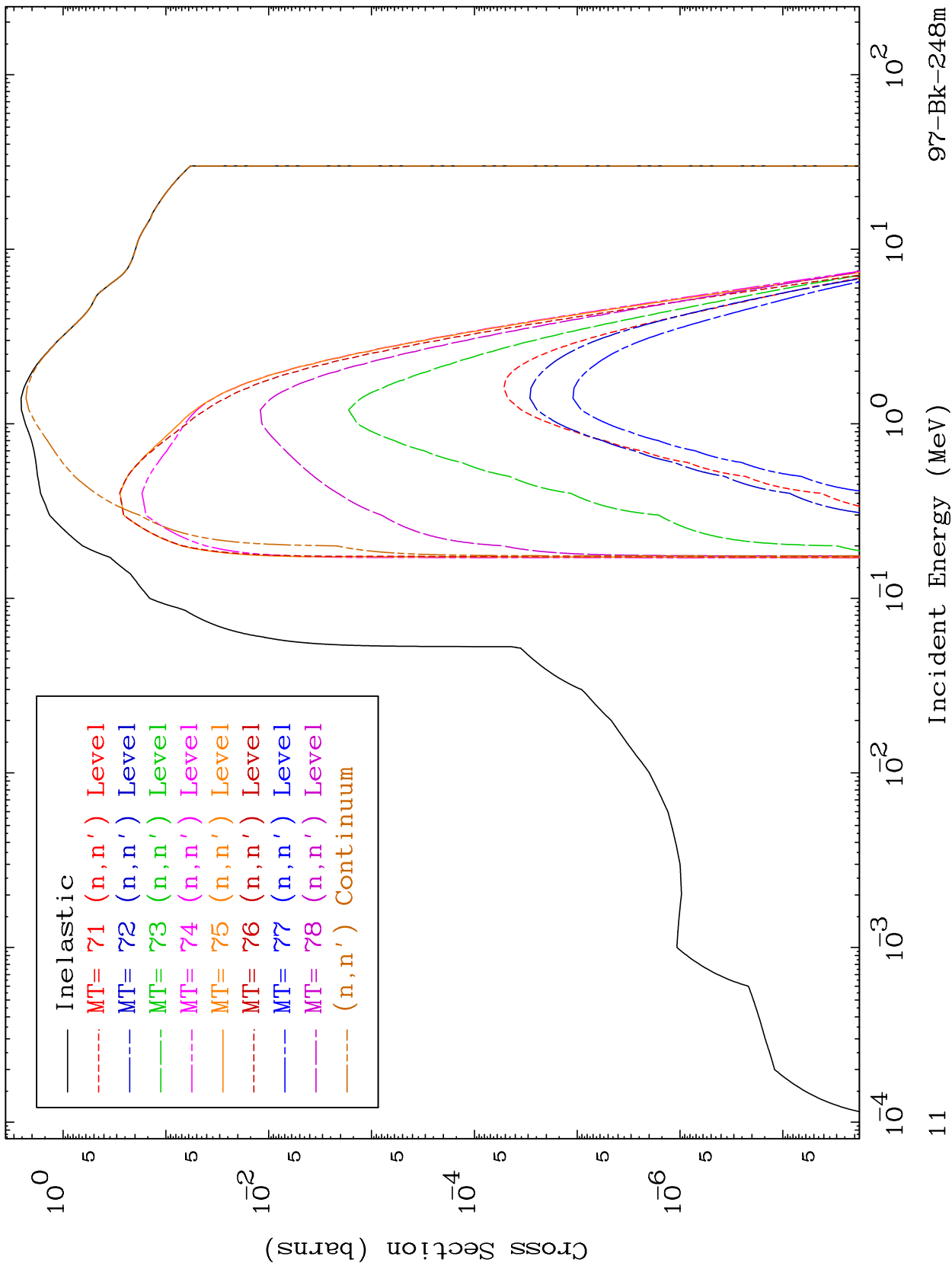
97-Bk-248m



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293 Kelvin Cross Sections

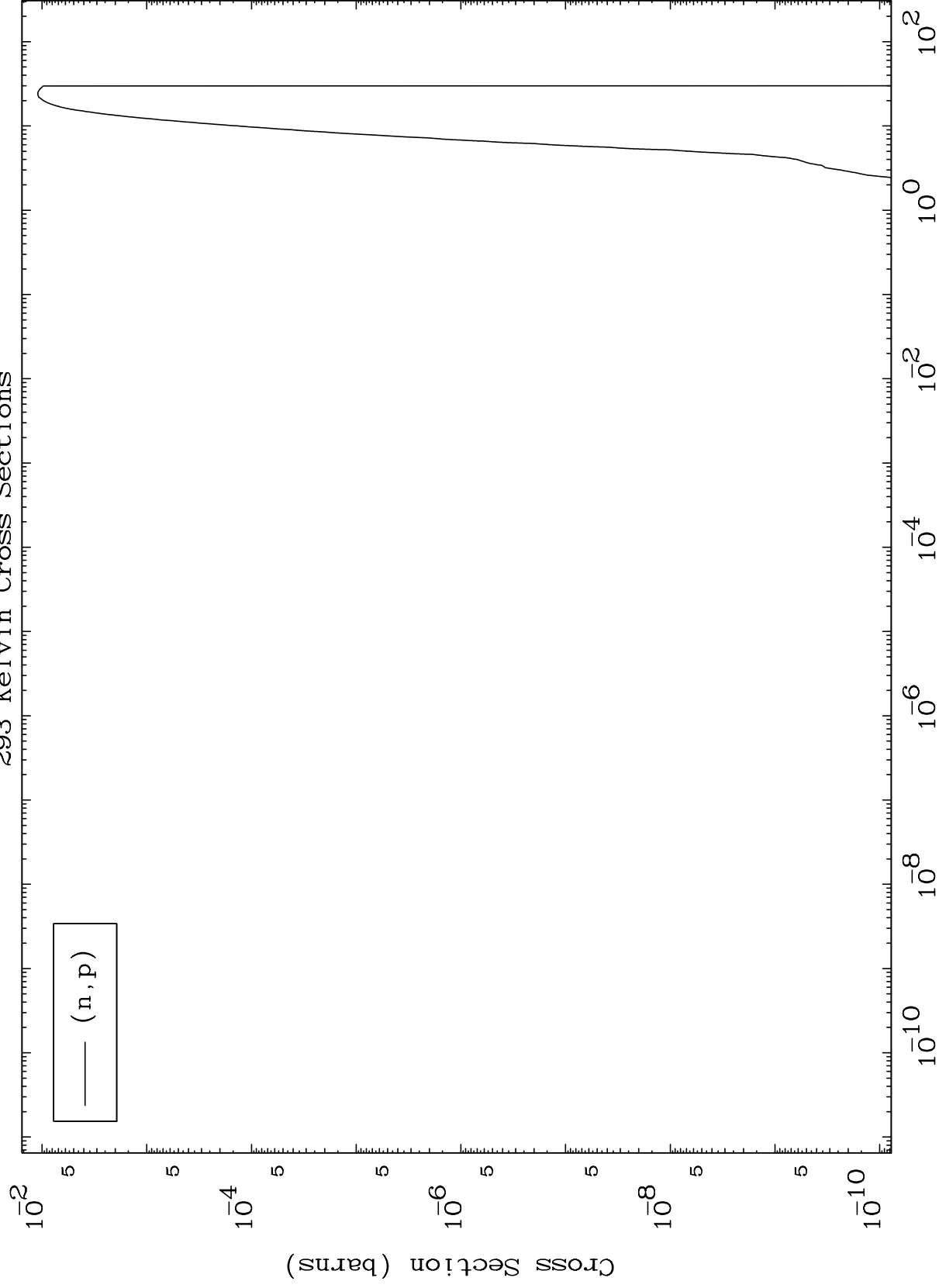
97-Bk-248m



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(n,p) Levels  
293 Kelvin Cross Sections

97-Bk-248m



12

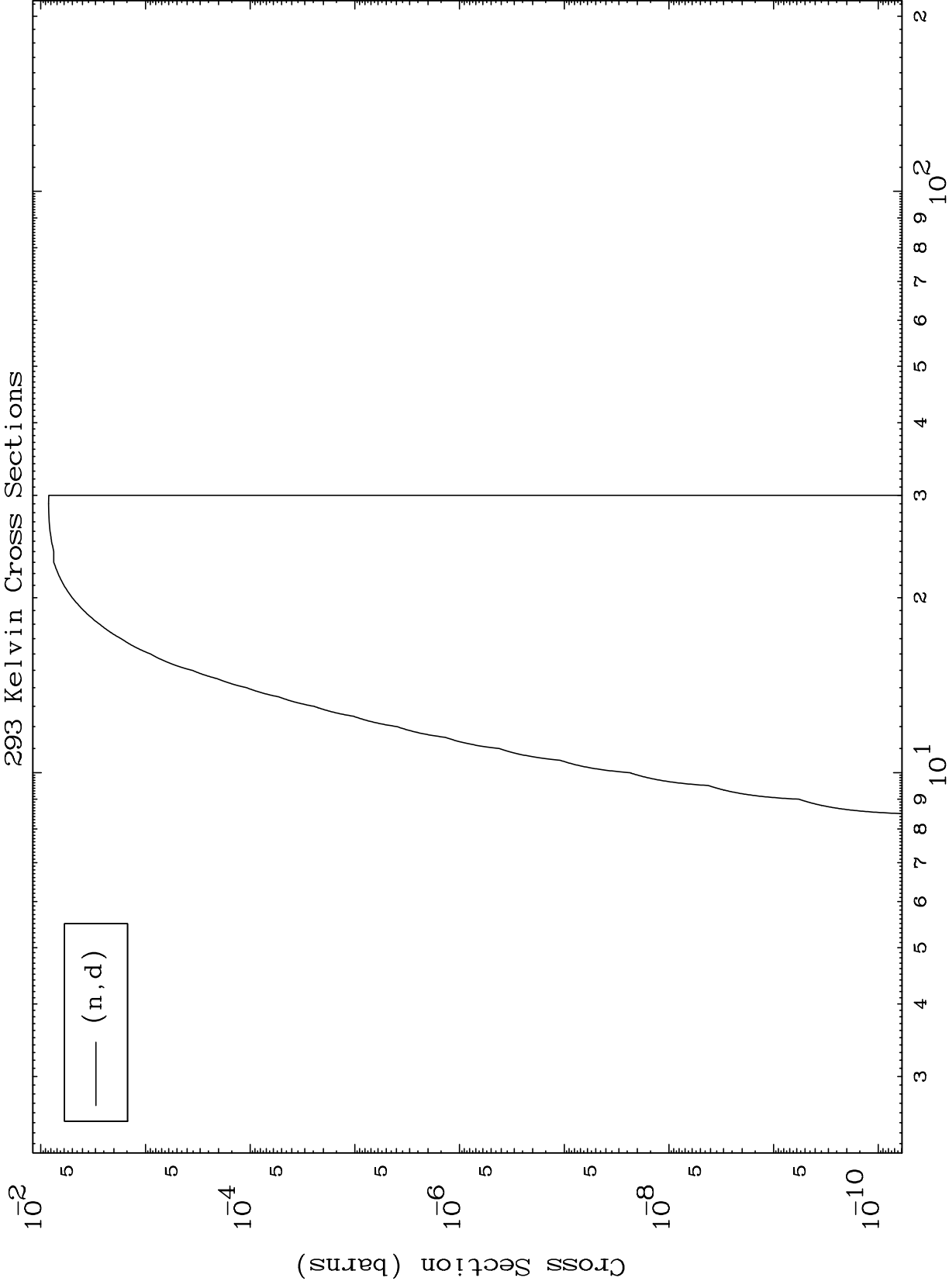
Incident Energy (MeV)

97-Bk-248m

MAT 9750

(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-248m



13

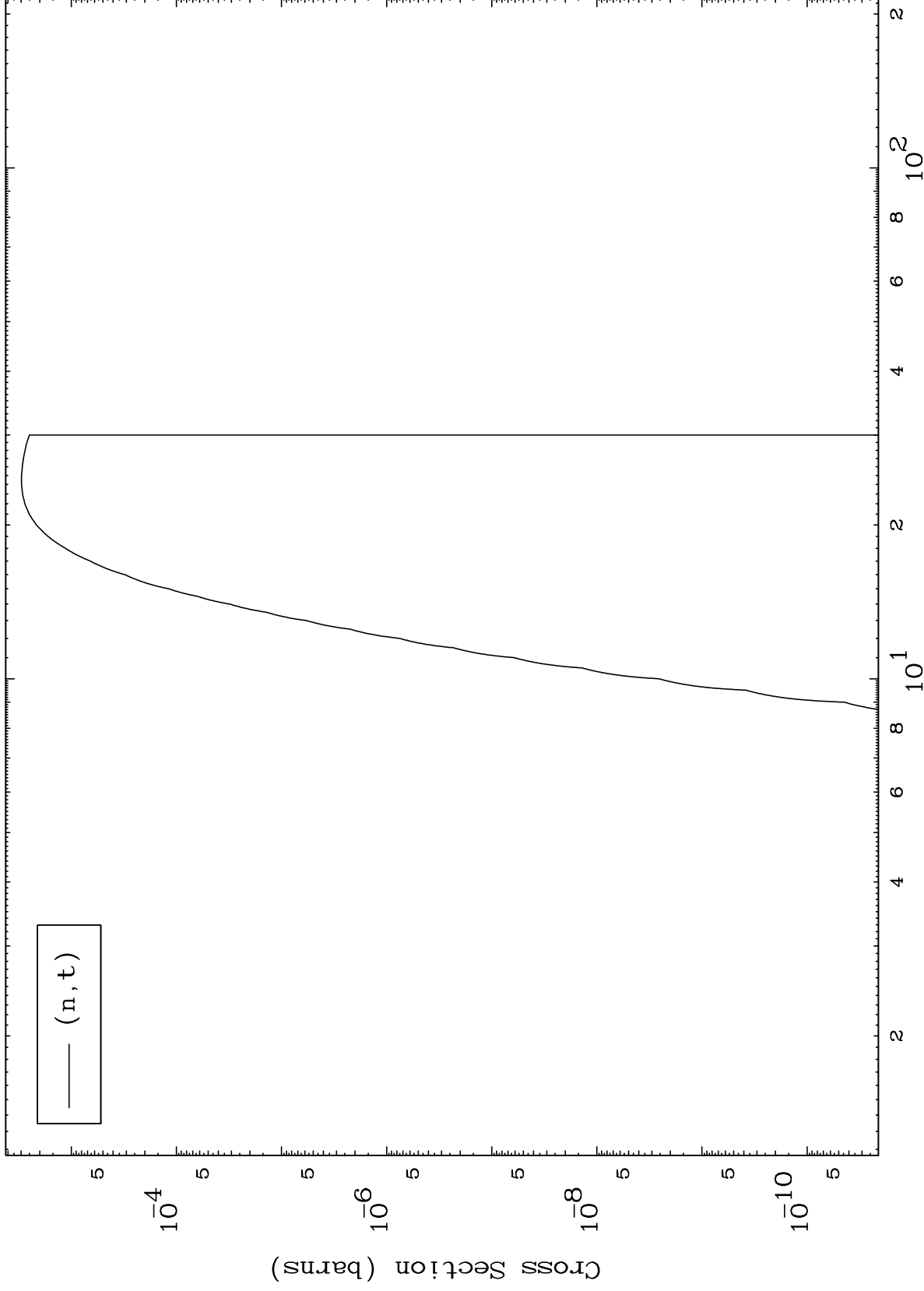
Incident Energy (MeV)

97-Bk-248m

MAT 9750

(n,t) Levels  
293 Kelvin Cross Sections

97-Bk-248m



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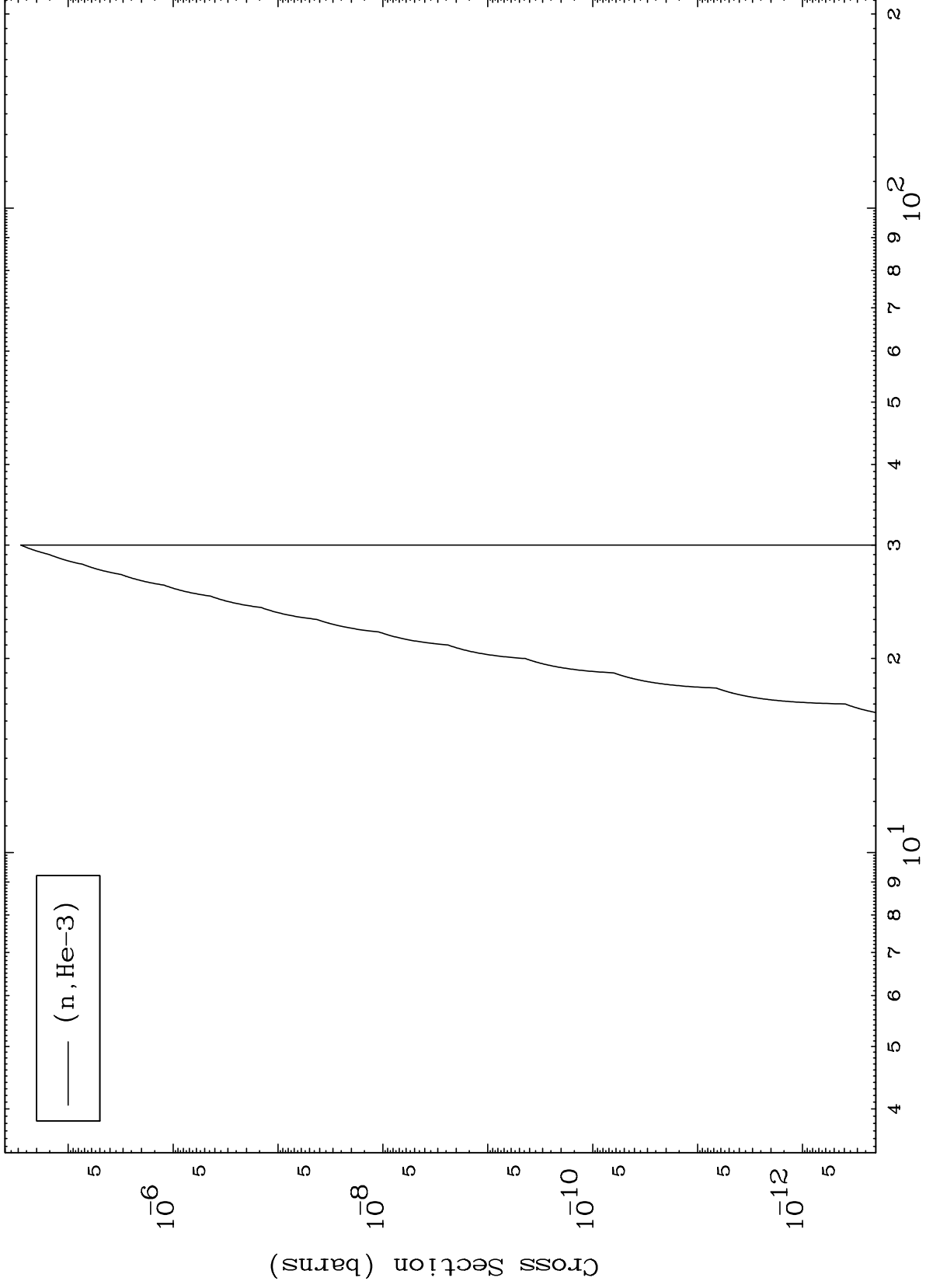
Incident Energy (MeV)

97-Bk-248m

MAT 9750

(n,He3) Levels  
293 Kelvin Cross Sections

97-Bk-248m



15

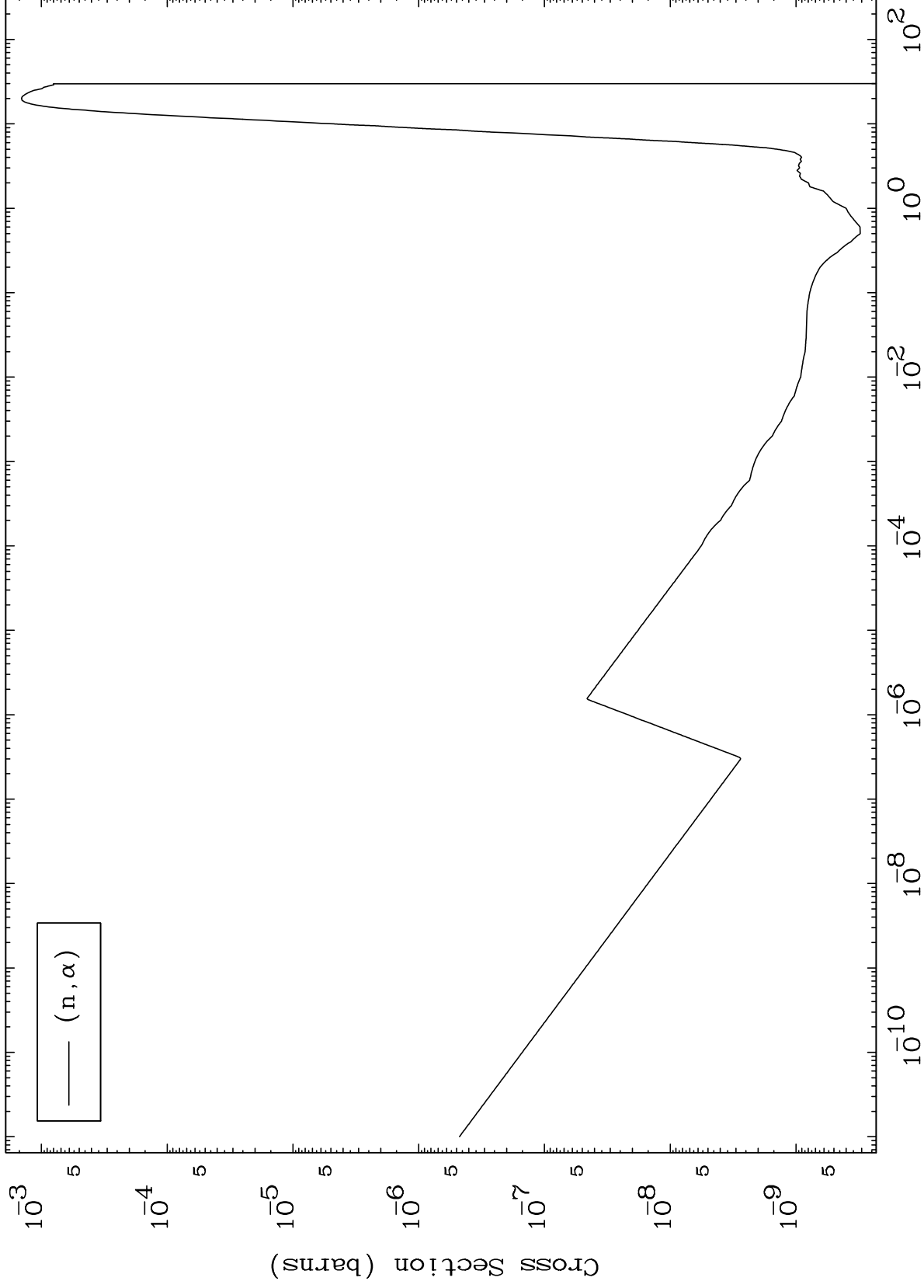
Incident Energy (MeV)

97-Bk-248m

MAT 9750

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

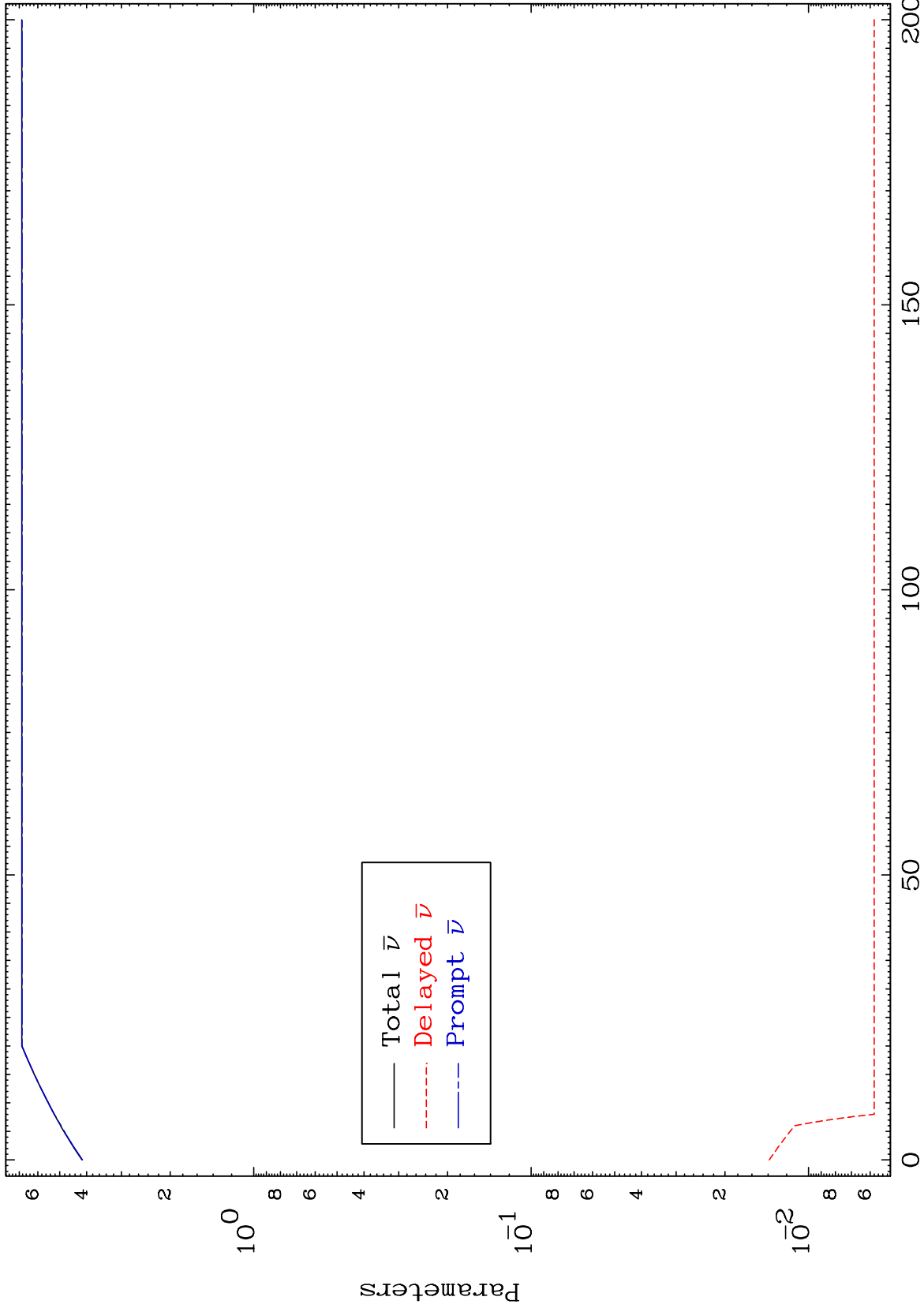
97-Bk-248m



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Energy Release  
Parameters

97-Bk-248m

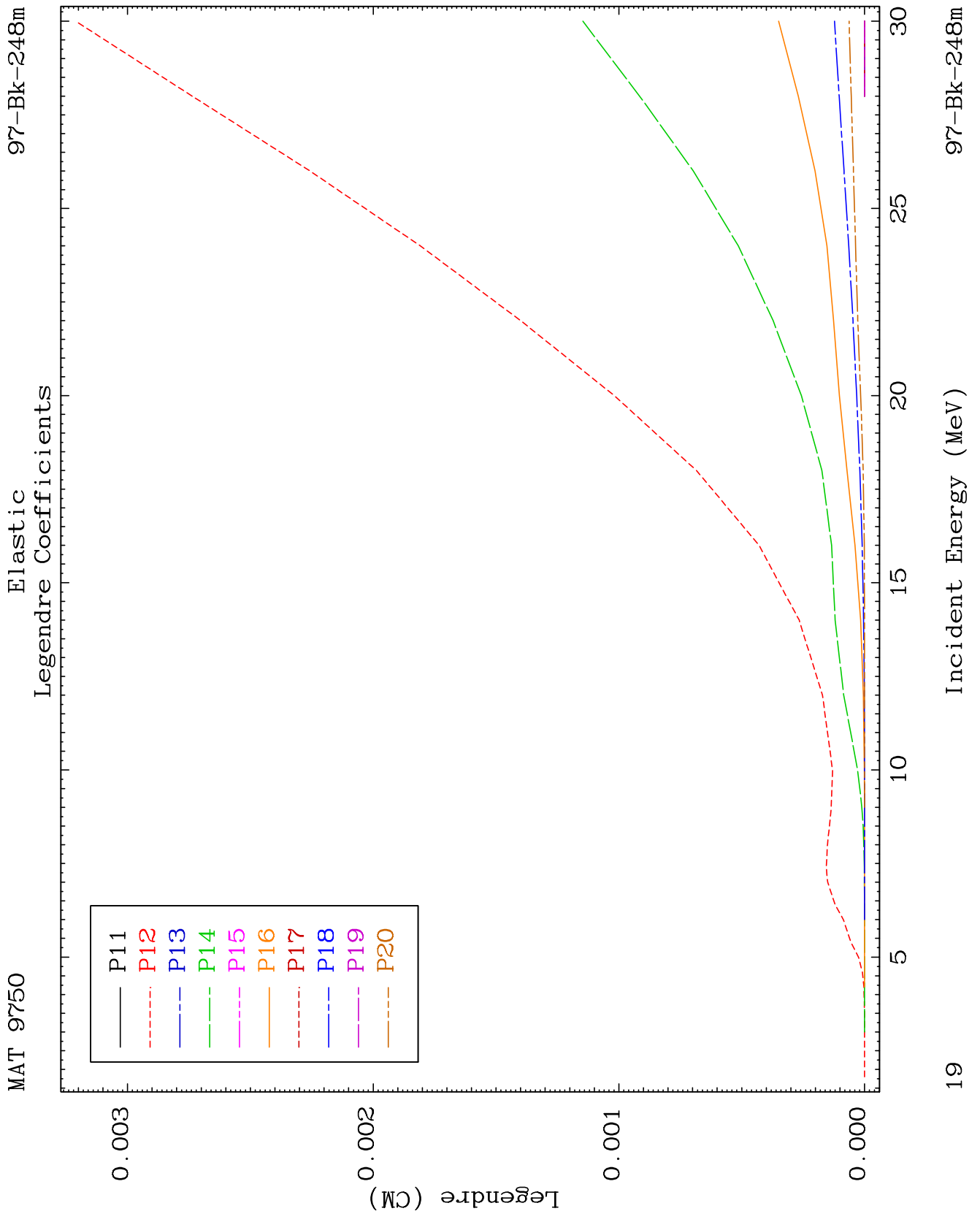


17

Incident Energy (MeV)

97-Bk-248m



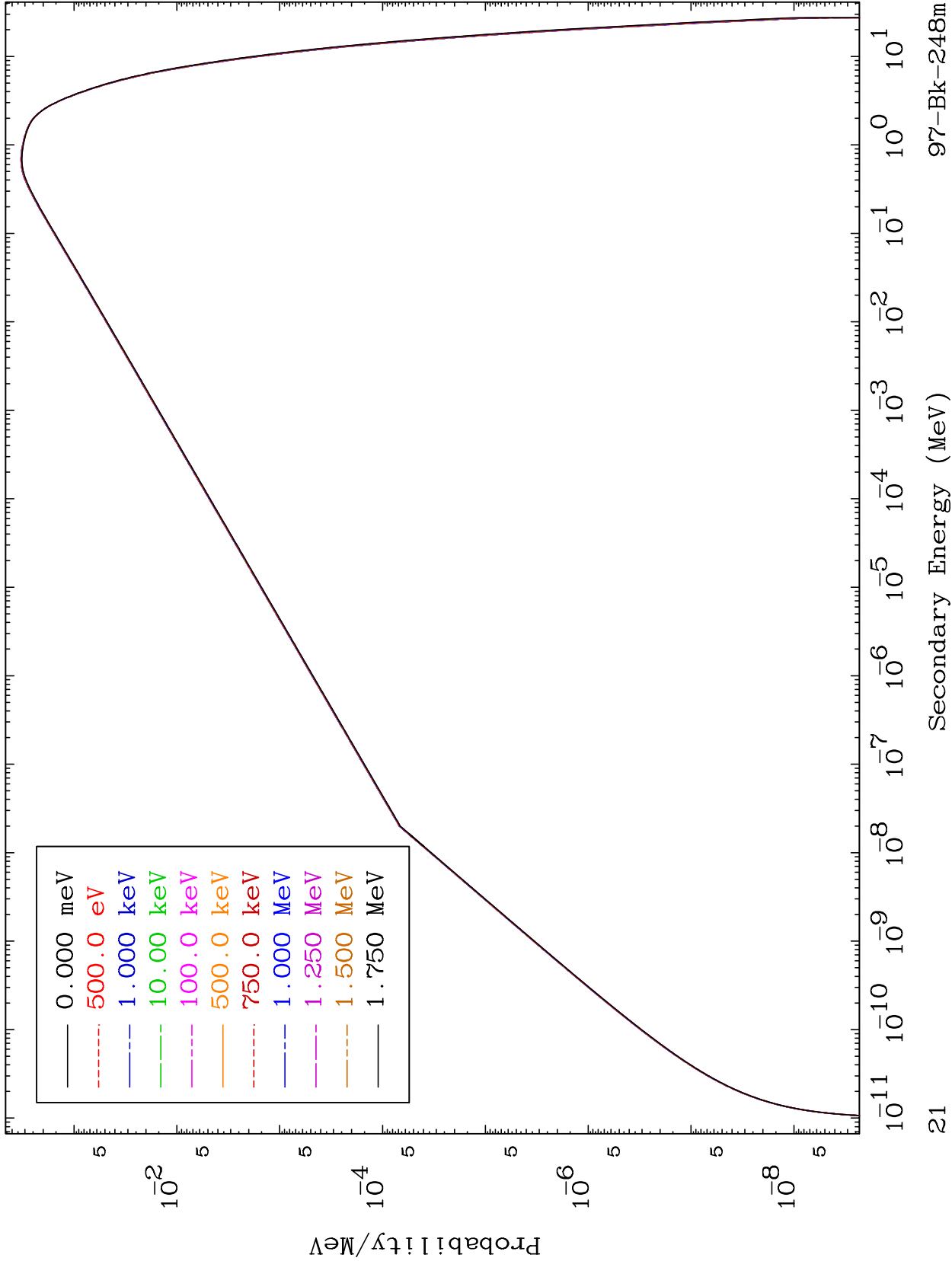




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### Fission Energy Distributions

97-Bk-248m

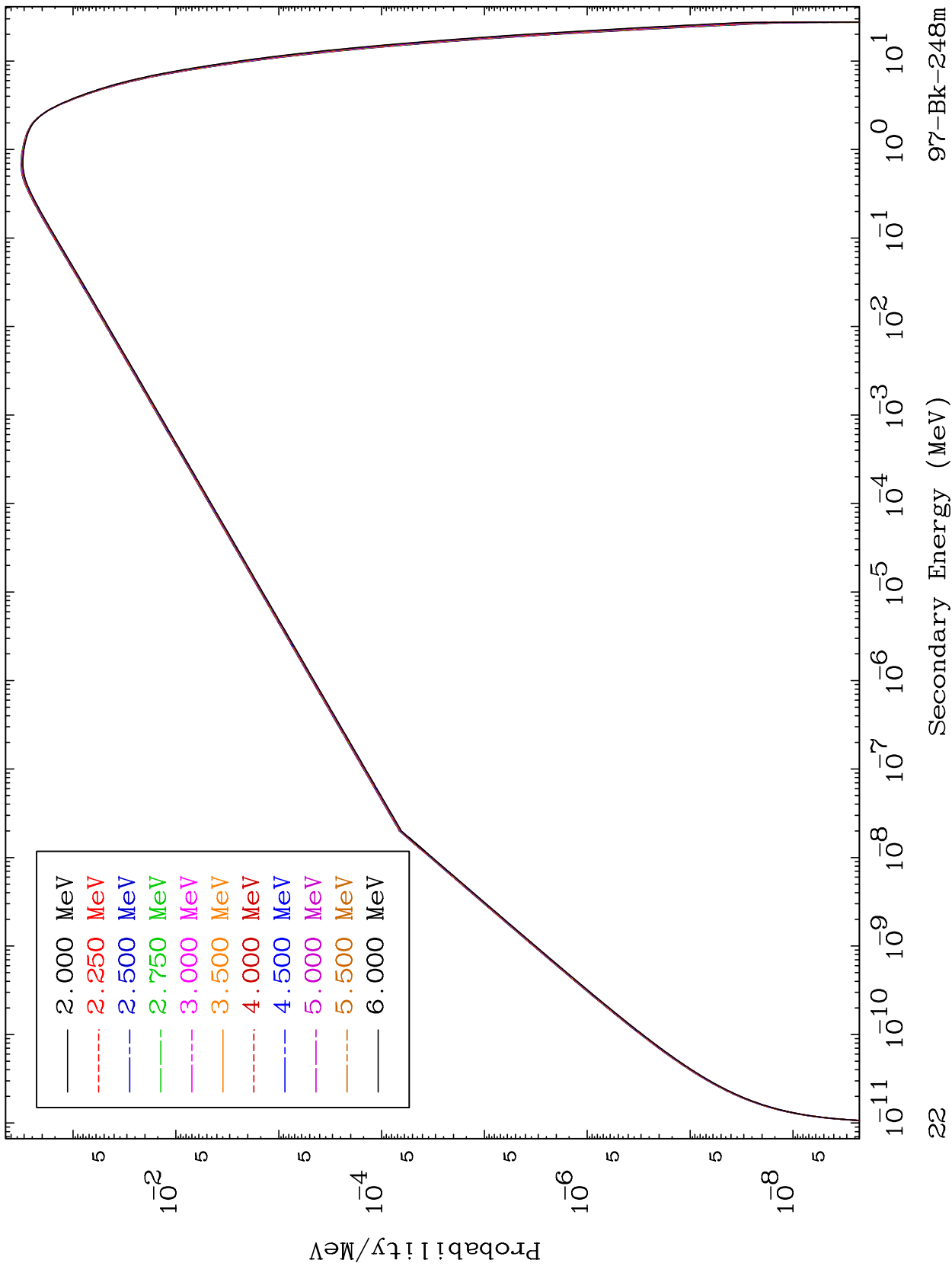


97-Bk-248m

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### Fission Energy Distributions

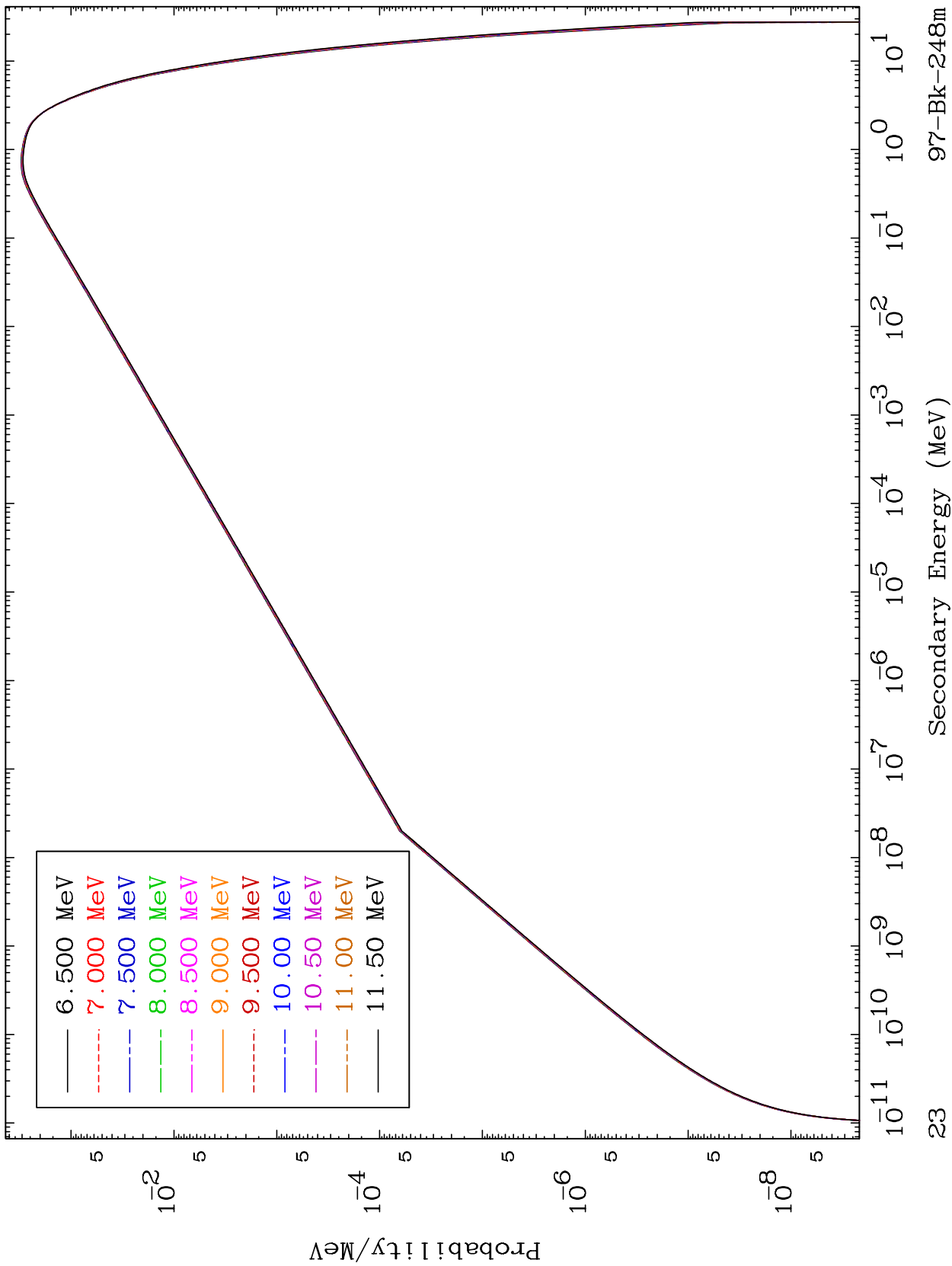
97-Bk-248m



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### Fission Energy Distributions

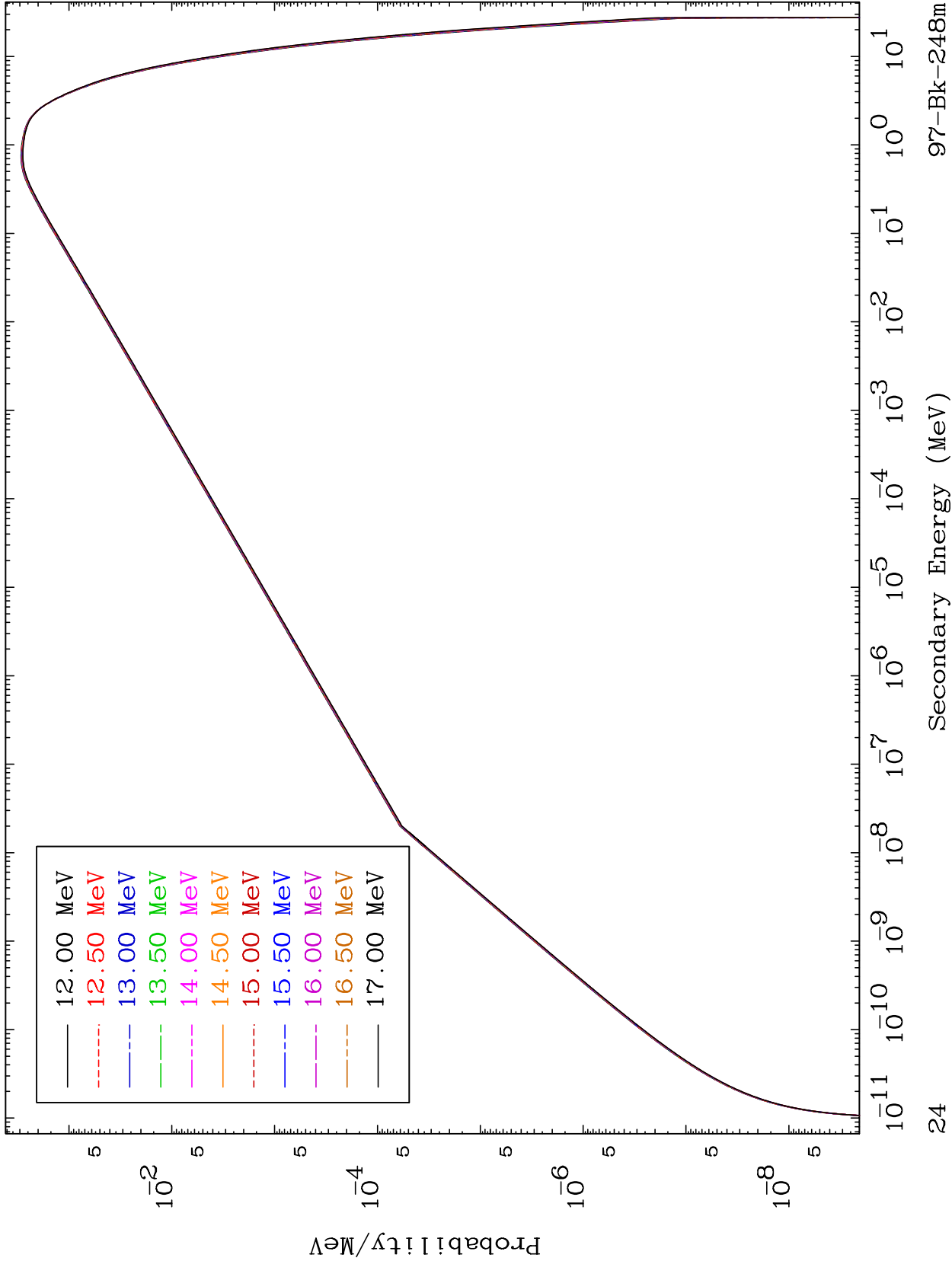
97-Bk-248m



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### Fission Energy Distributions

97-Bk-248m



97-Bk-248m

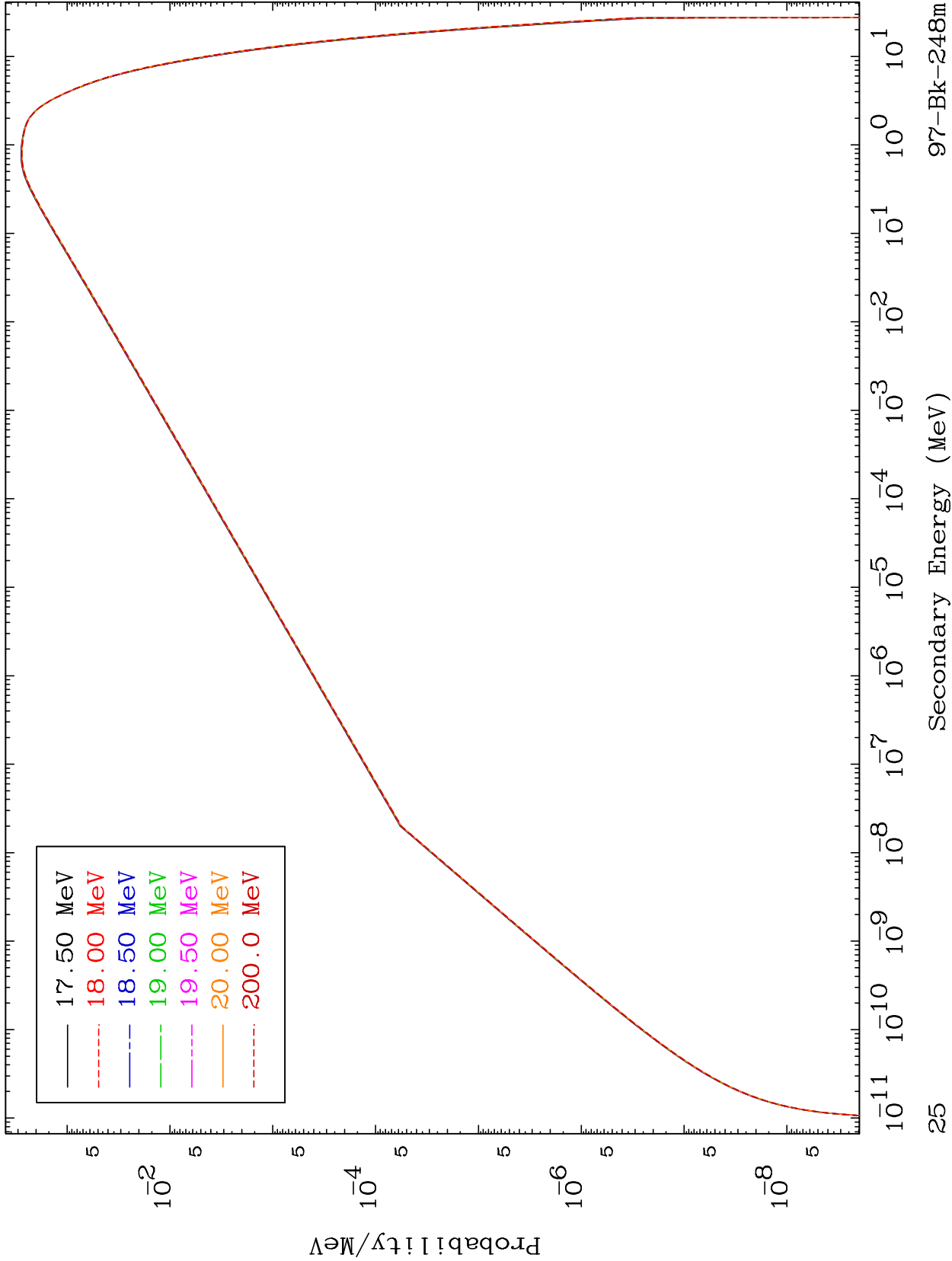
Secondary Energy (MeV)

24

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### Fission Energy Distributions

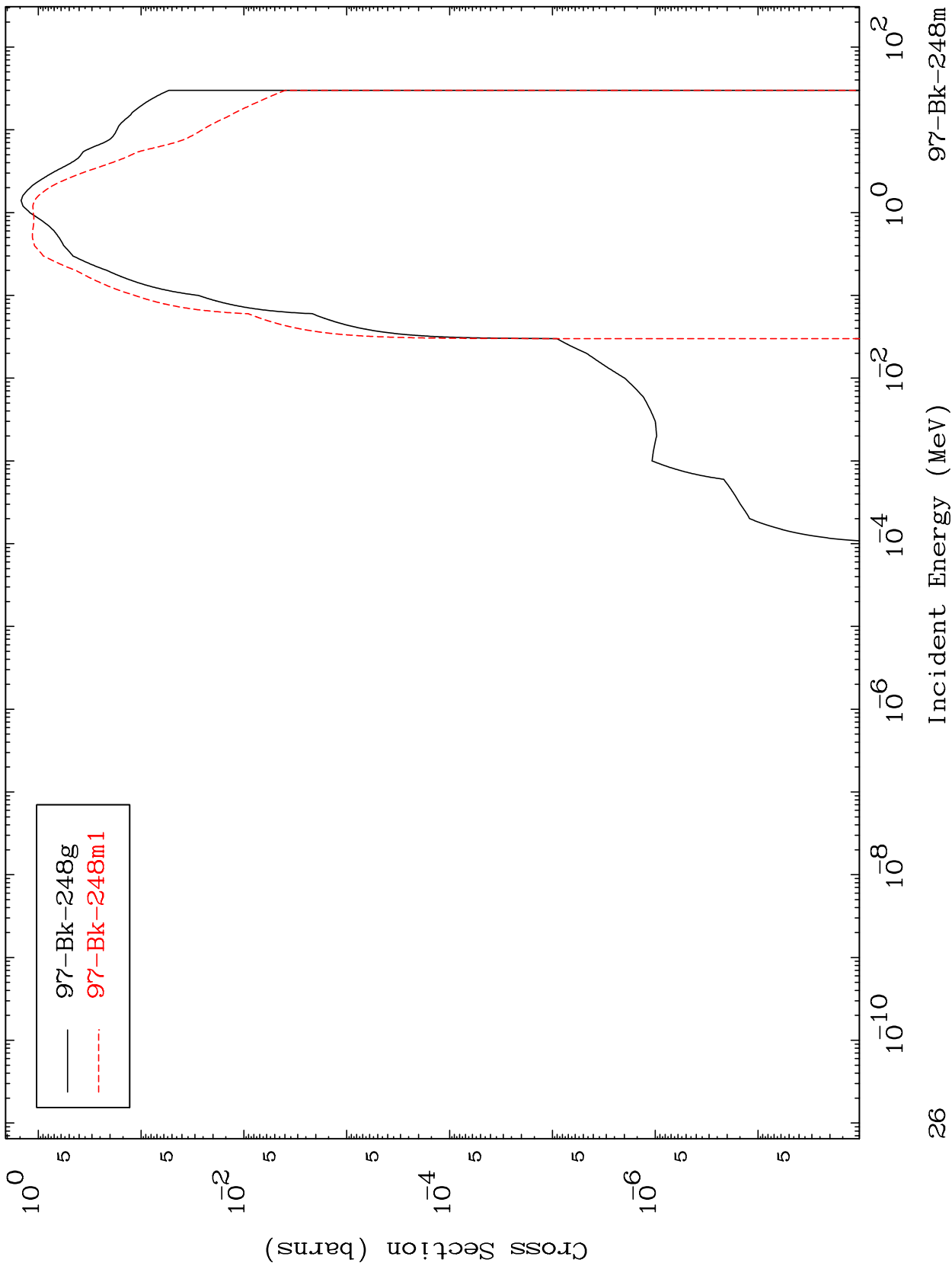
97-Bk-248m



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97-Bk-248m

Inelastic  
Radionuclide Production Cross Section

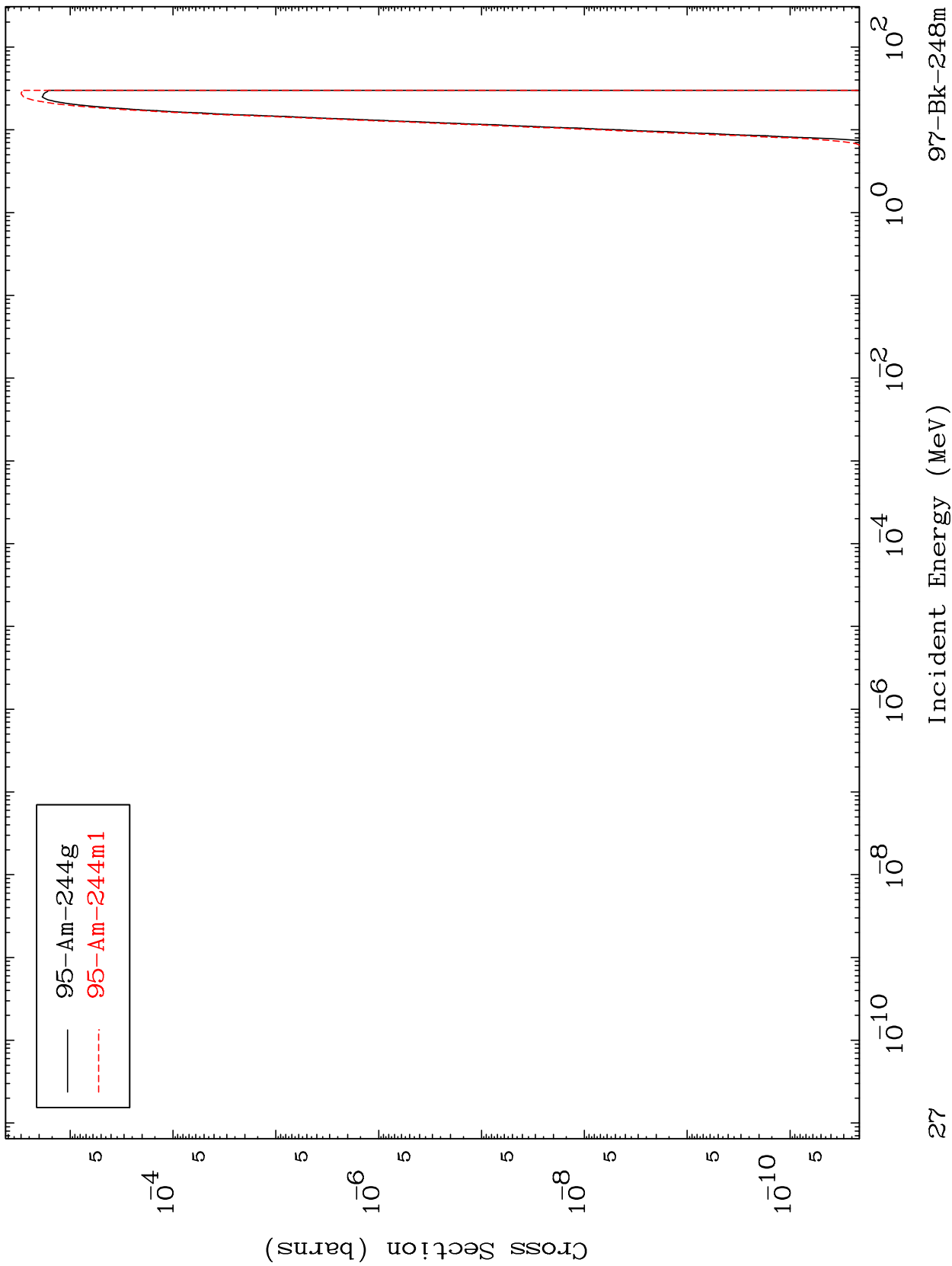


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(n,n')  $\alpha$

97-Bk-248m

Radionuclide Production Cross Section

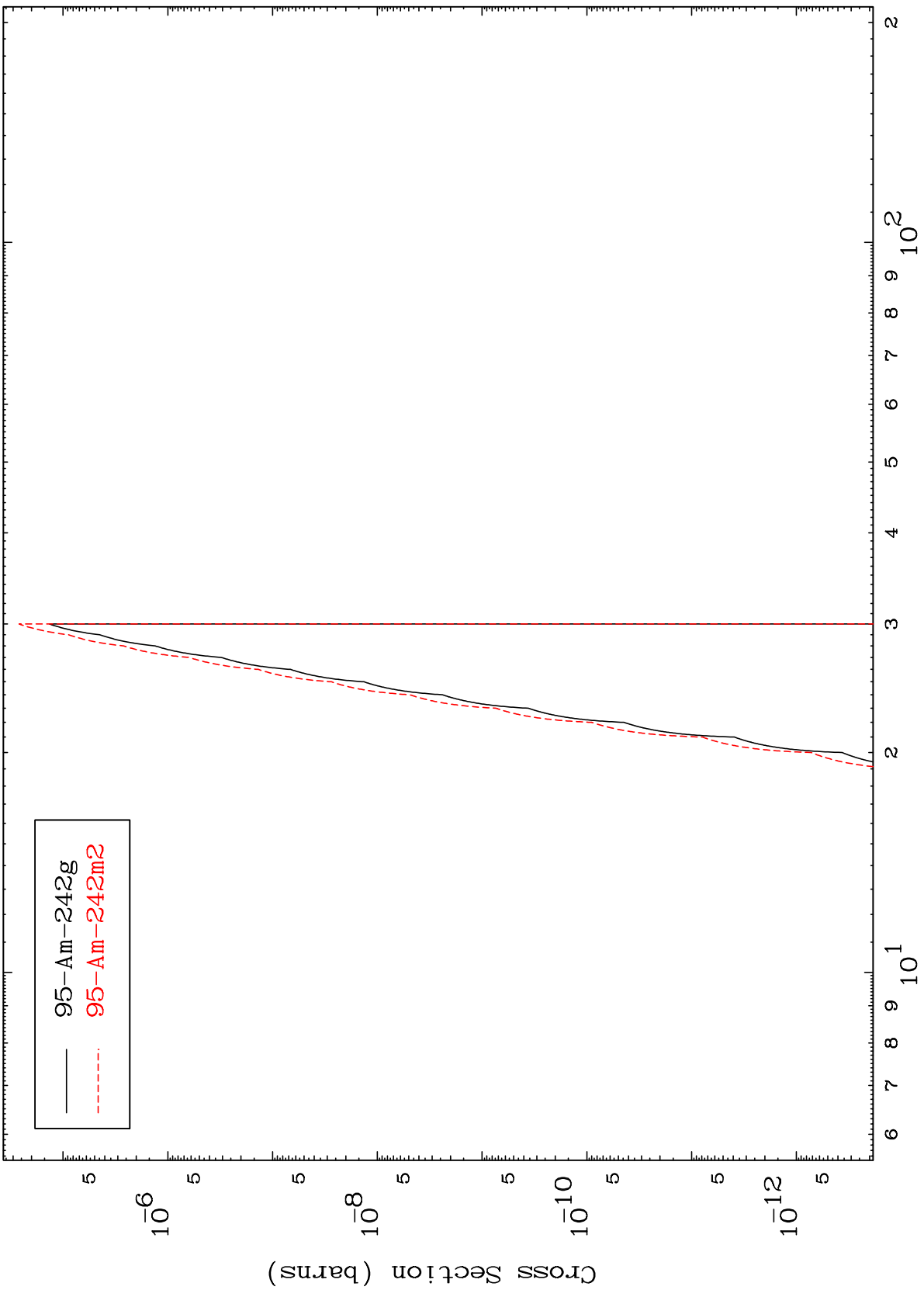


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(n,3n)  $\alpha$

97-Bk-248m

Radionuclide Production Cross Section



95-Am-242g  
95-Am-242m2

Incident Energy (MeV)

97-Bk-248m

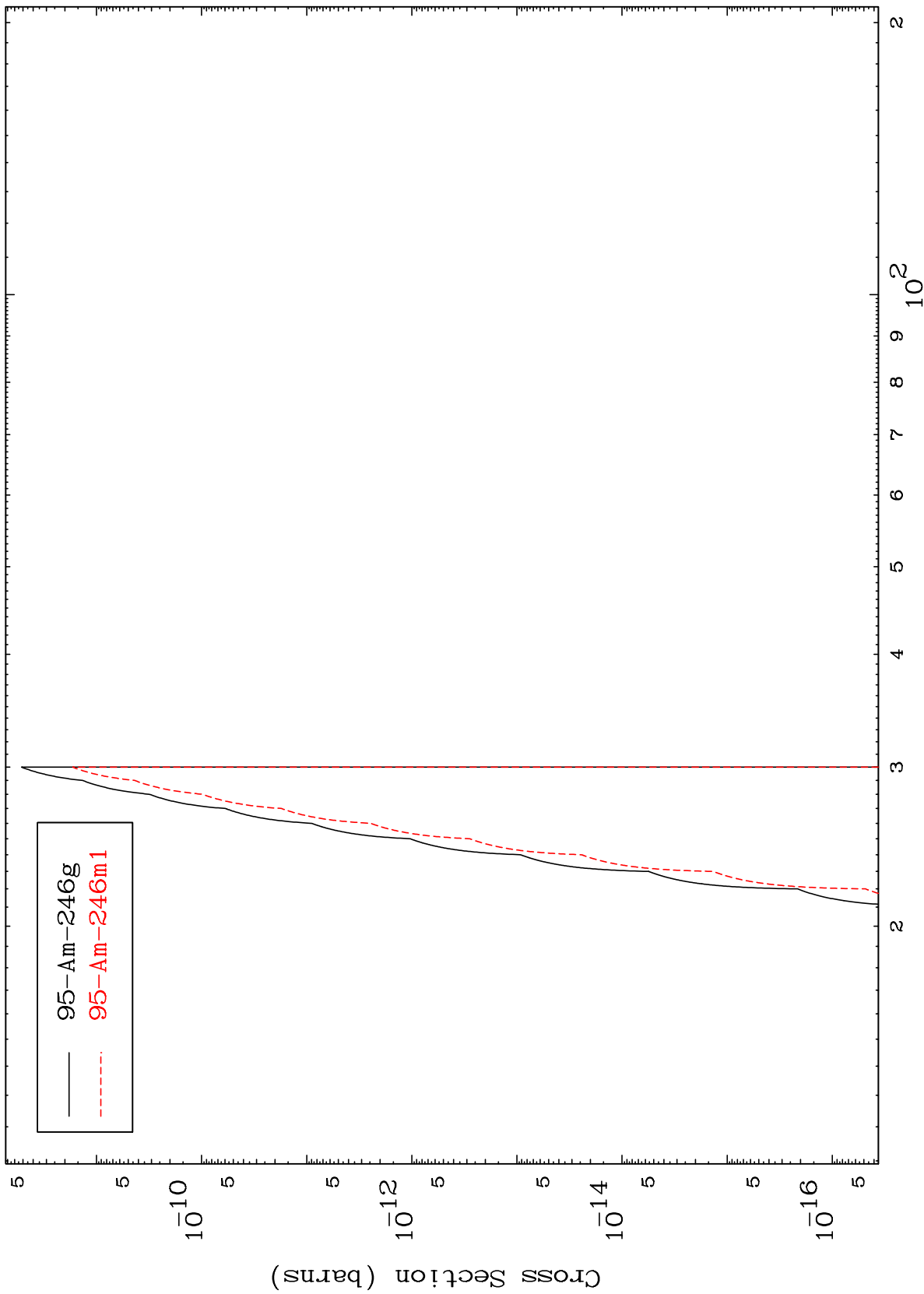
28

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(n,2n) p

97-Bk-248m

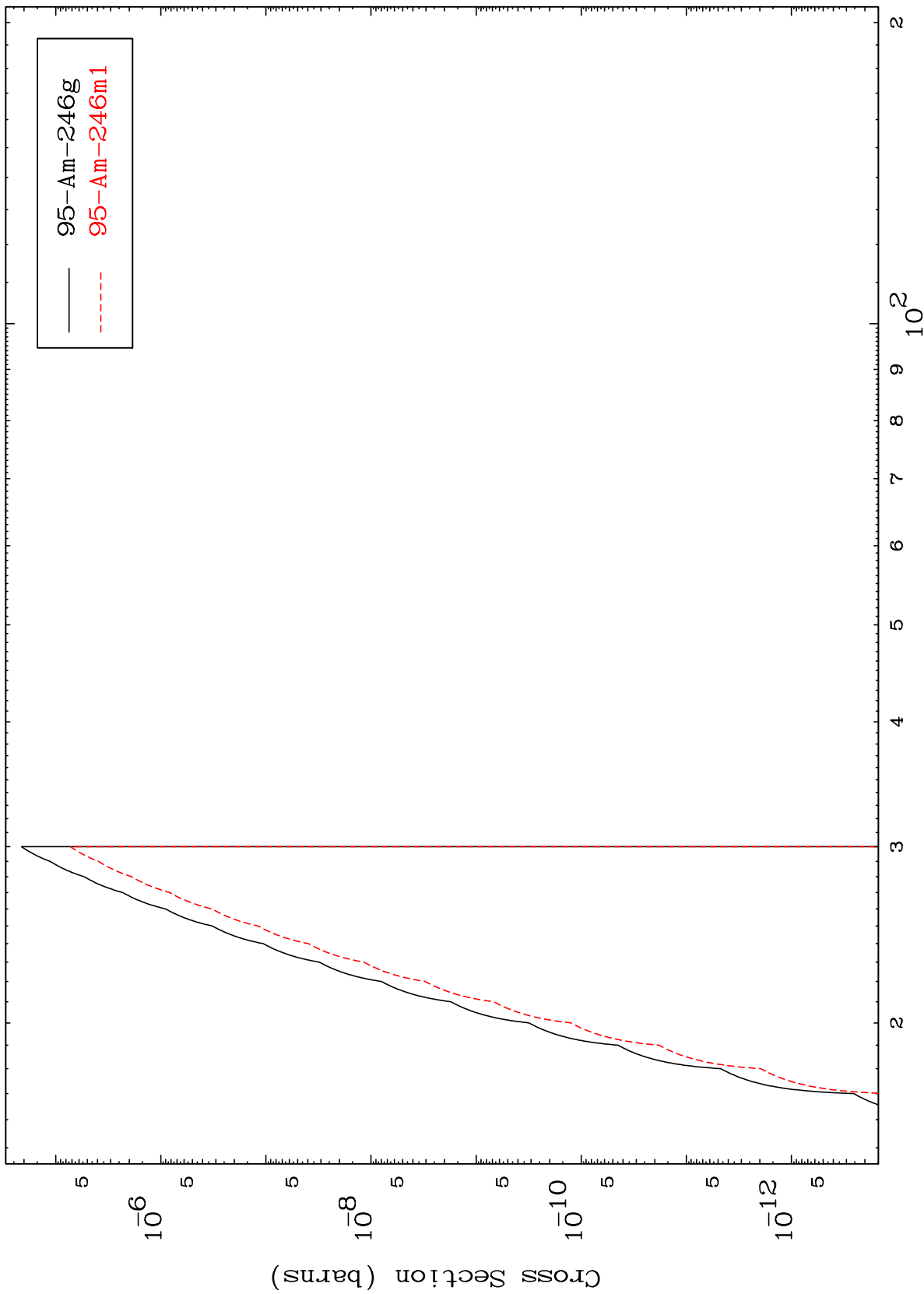
Radionuclide Production Cross Section



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97-Bk-248m

(n,He-3)  
Radionuclide Production Cross Section



30

Incident Energy (MeV)

97-Bk-248m