

Program EVALPLOT
(Version 2021-1)

by

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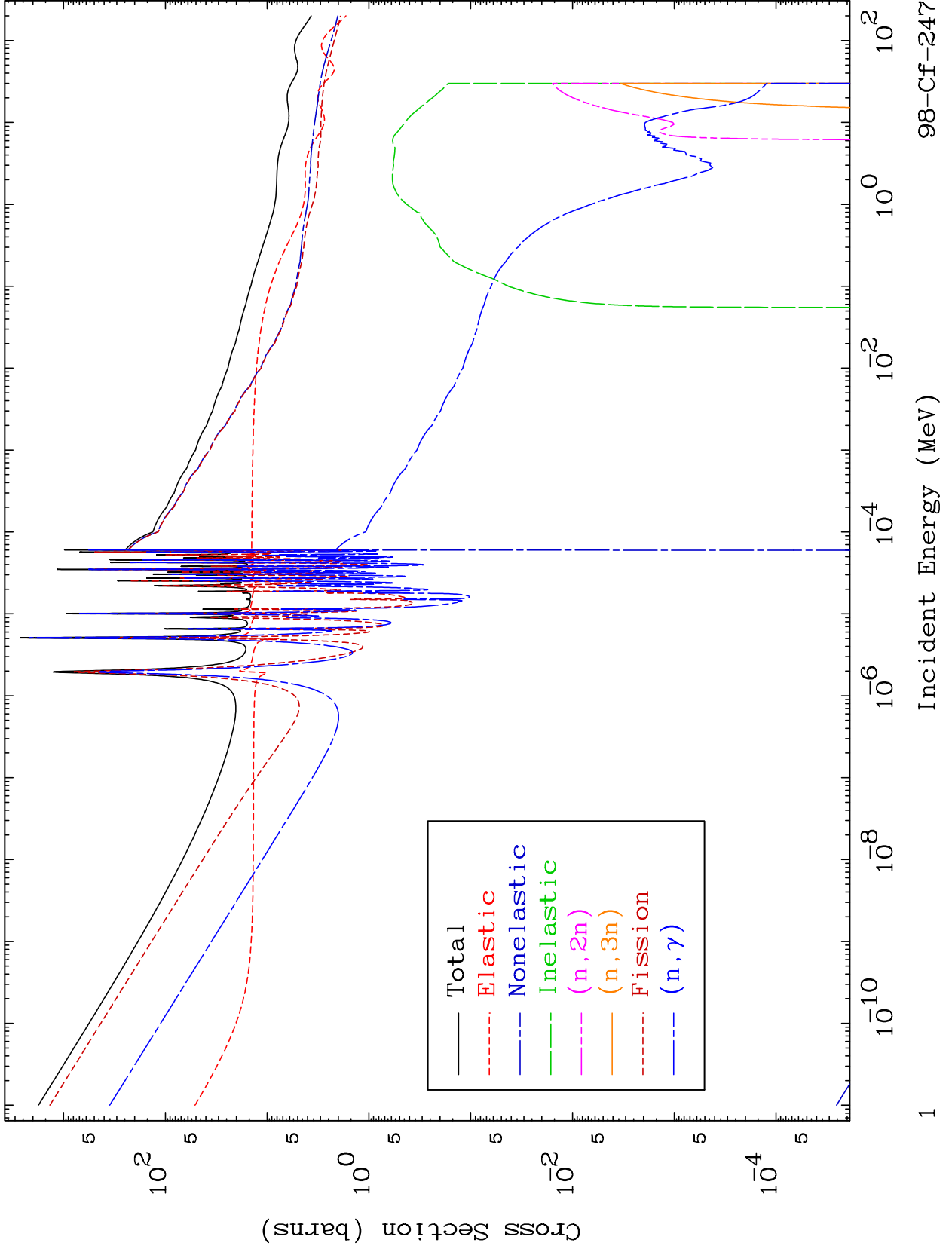
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Press Mouse Button to Start

MAT 9846

Neutron Major
293 Kelvin Cross Sections

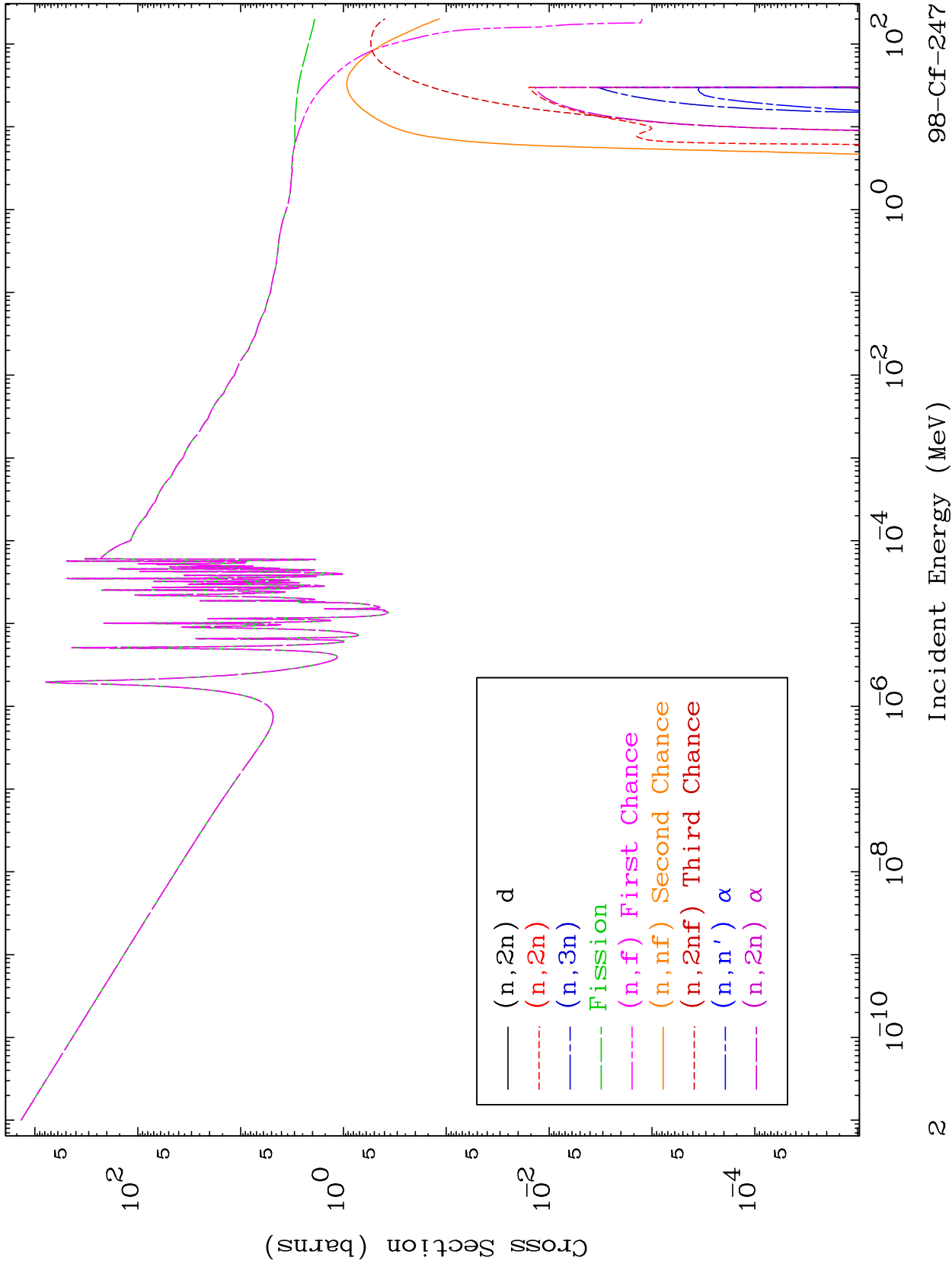
98-Cf-247



MAT 9846

Neutron Absorption
293 Kelvin Cross Sections

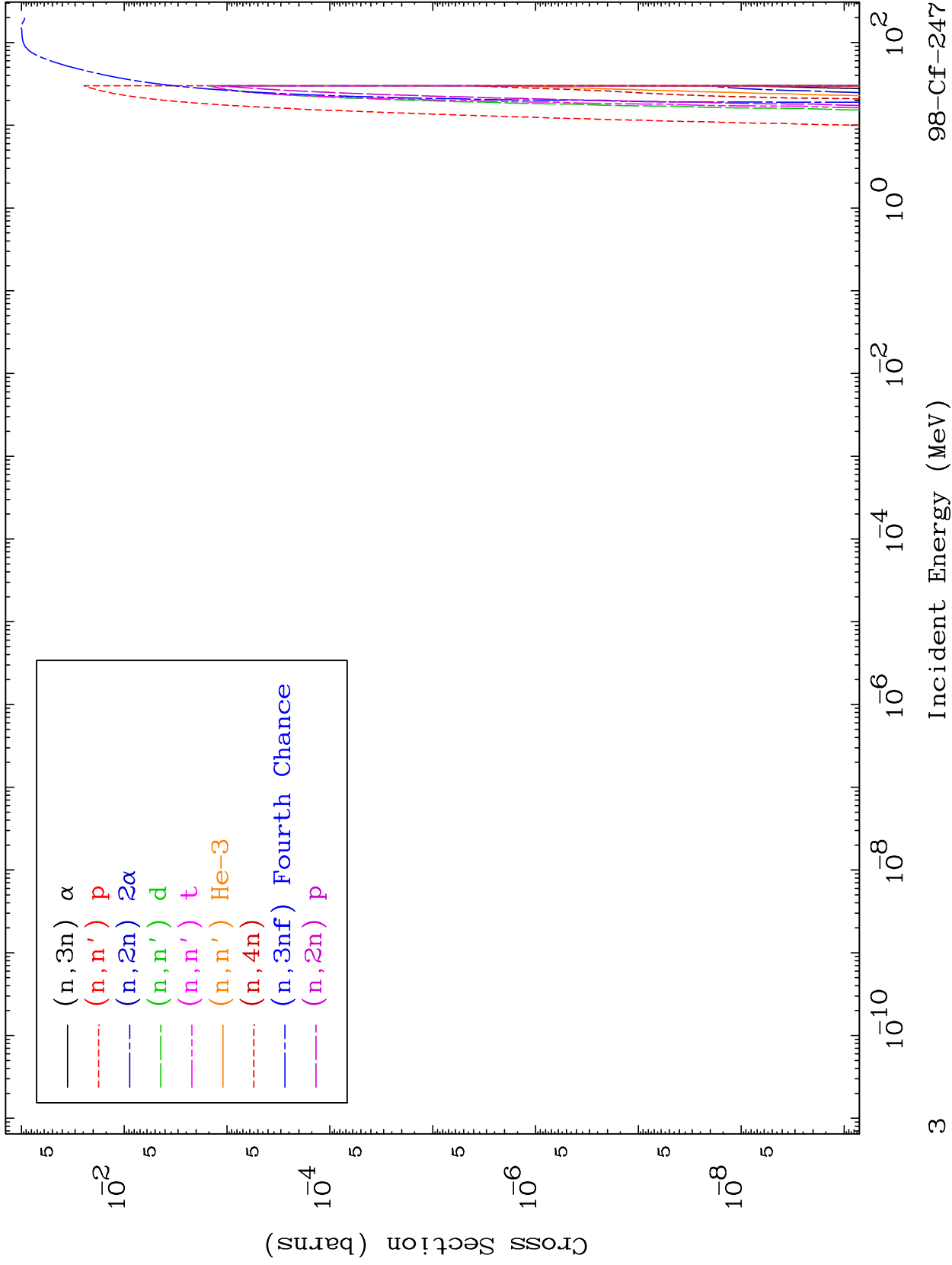
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MAT 9846

Neutron Absorption
293 Kelvin Cross Sections

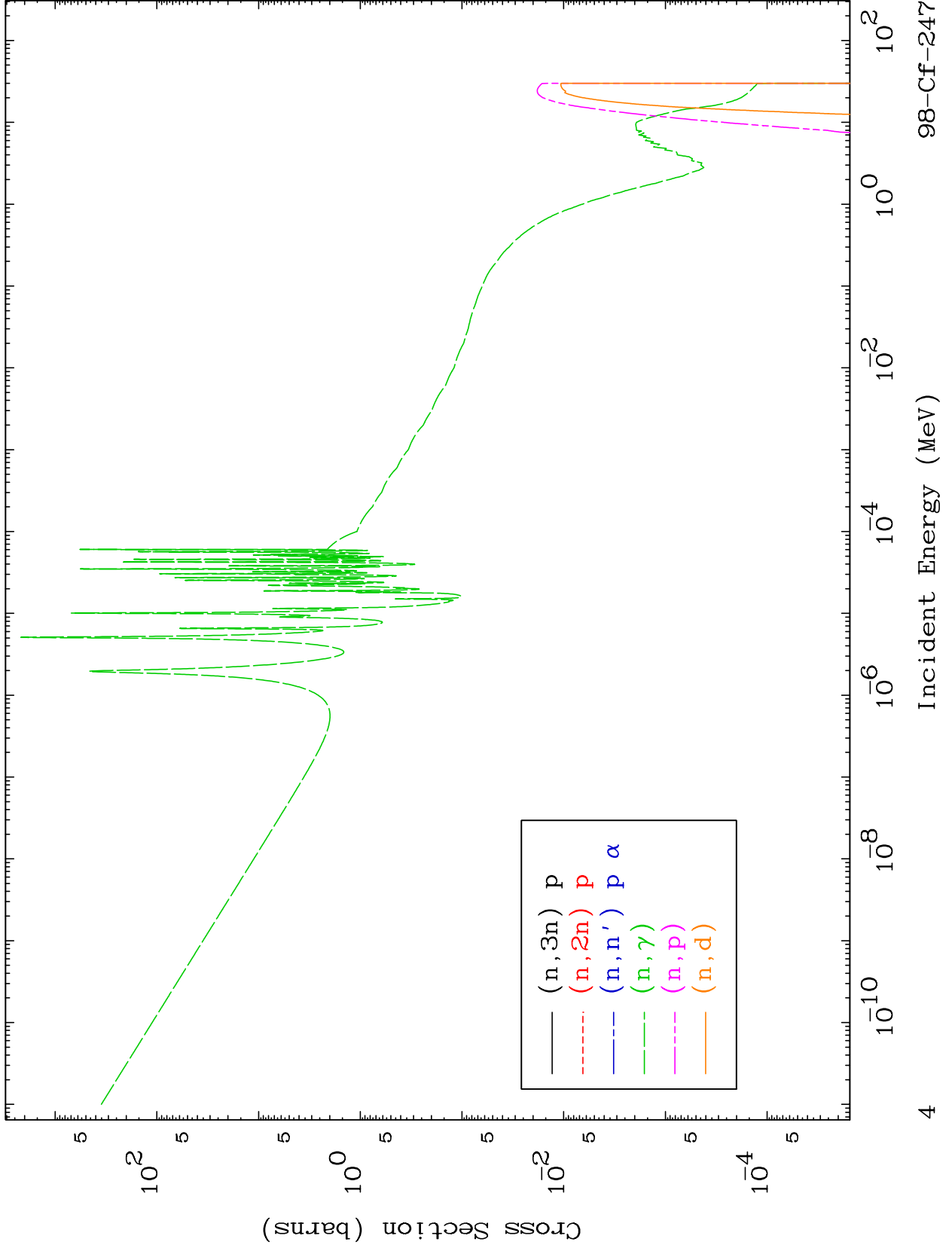
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Neutron Absorption
293 Kelvin Cross Sections

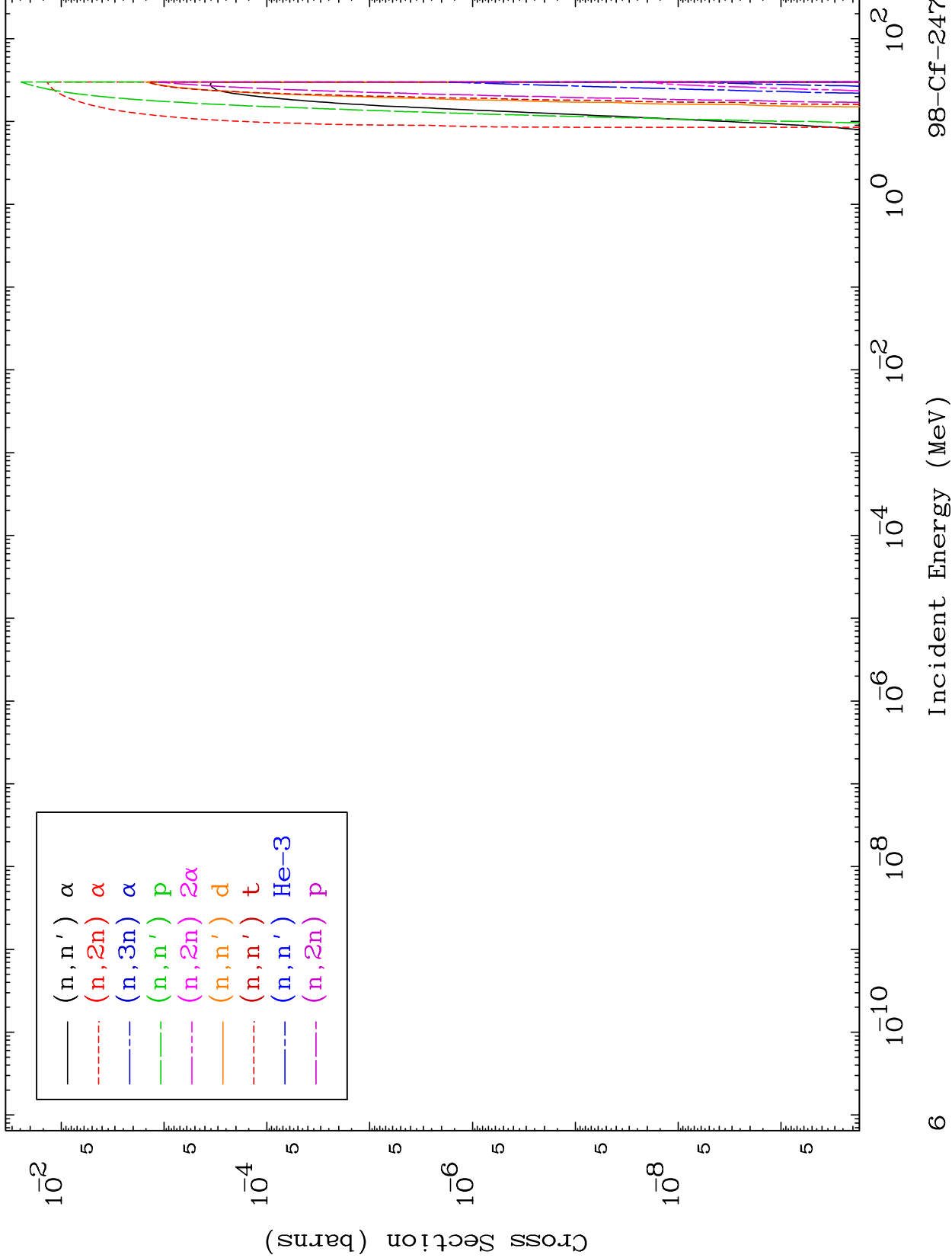
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MAT 9846

Charged Particle
293 Kelvin Cross Sections

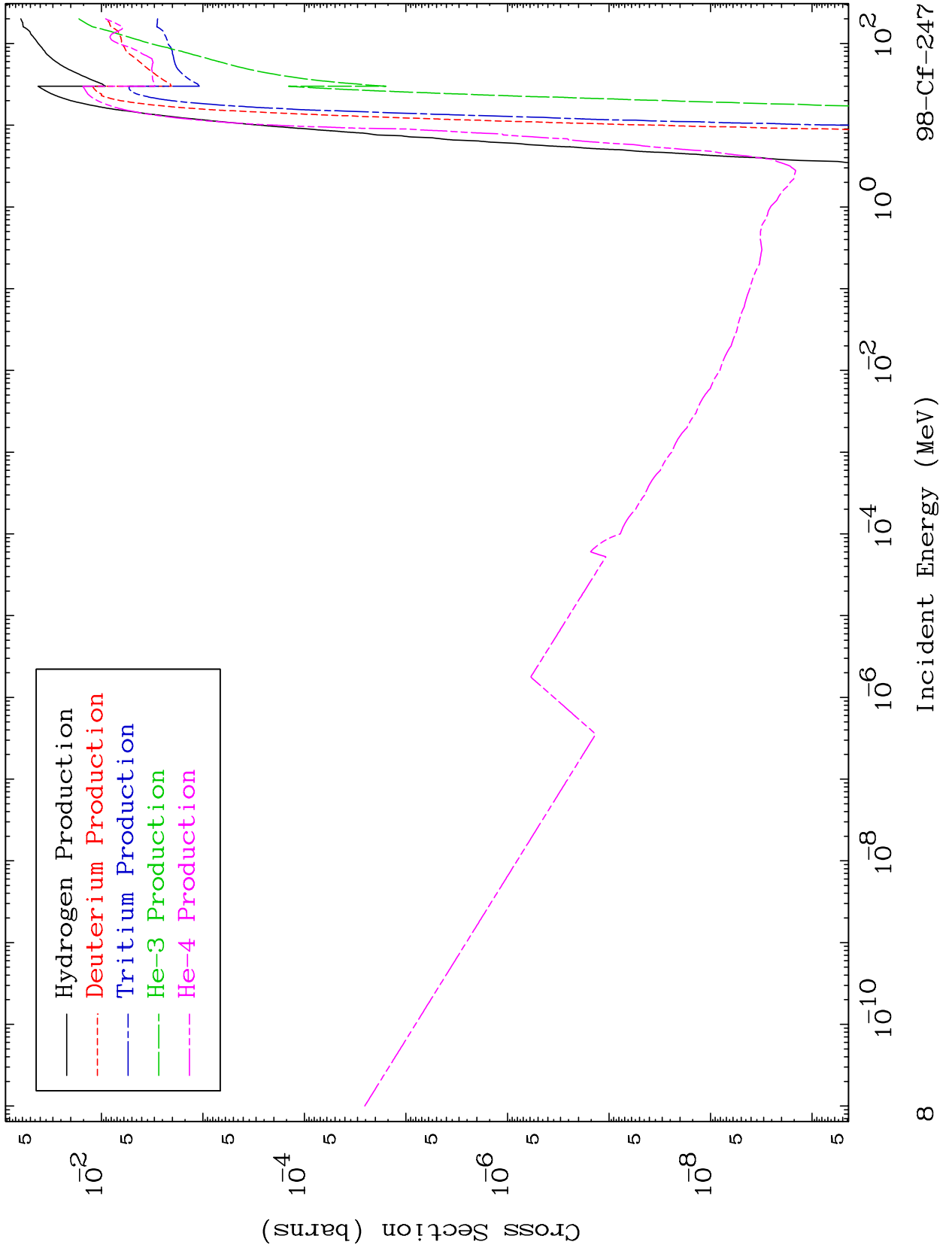
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Particle Production
293 Kelvin Cross Sections

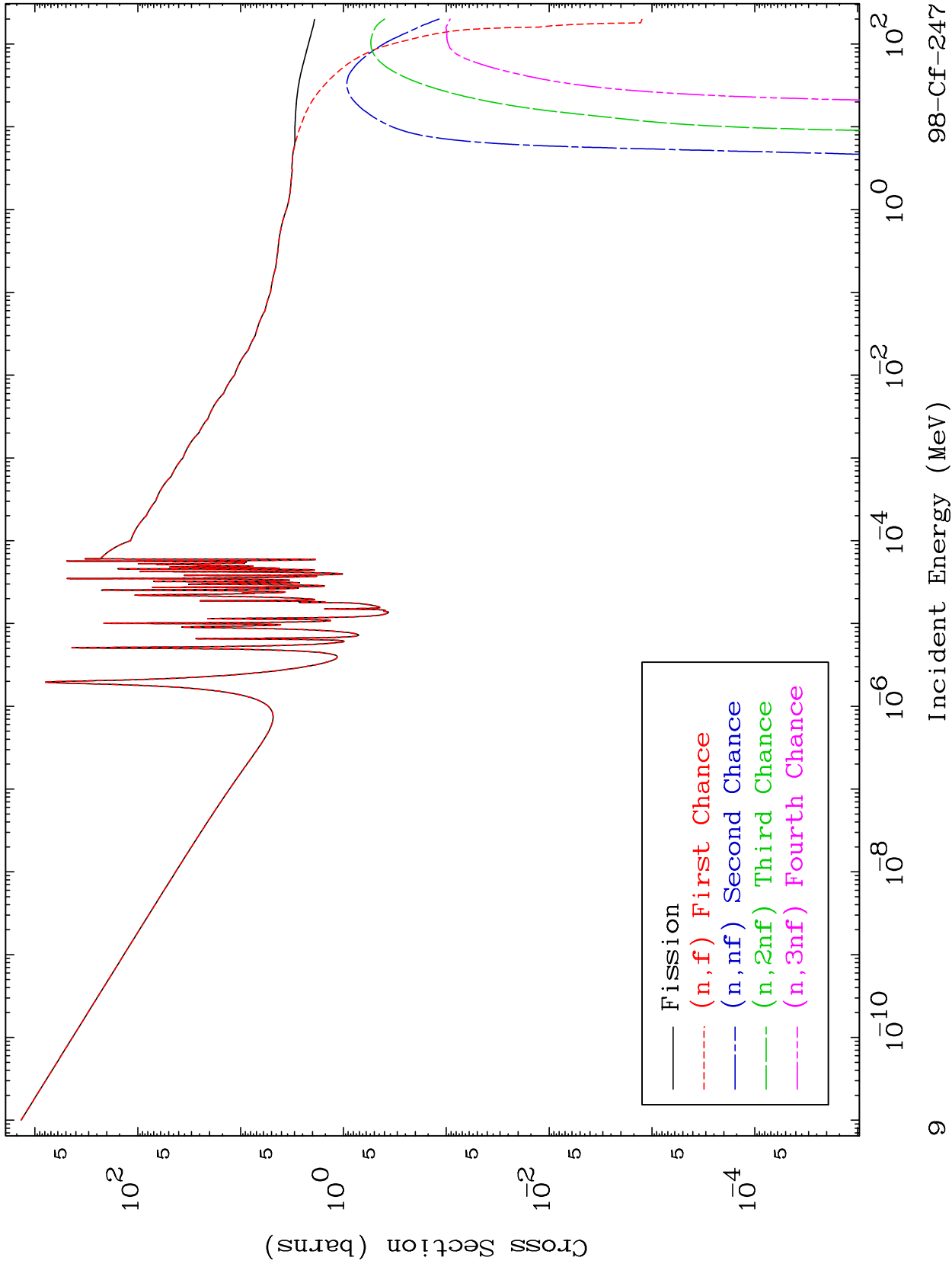
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Fission
293 Kelvin Cross Sections

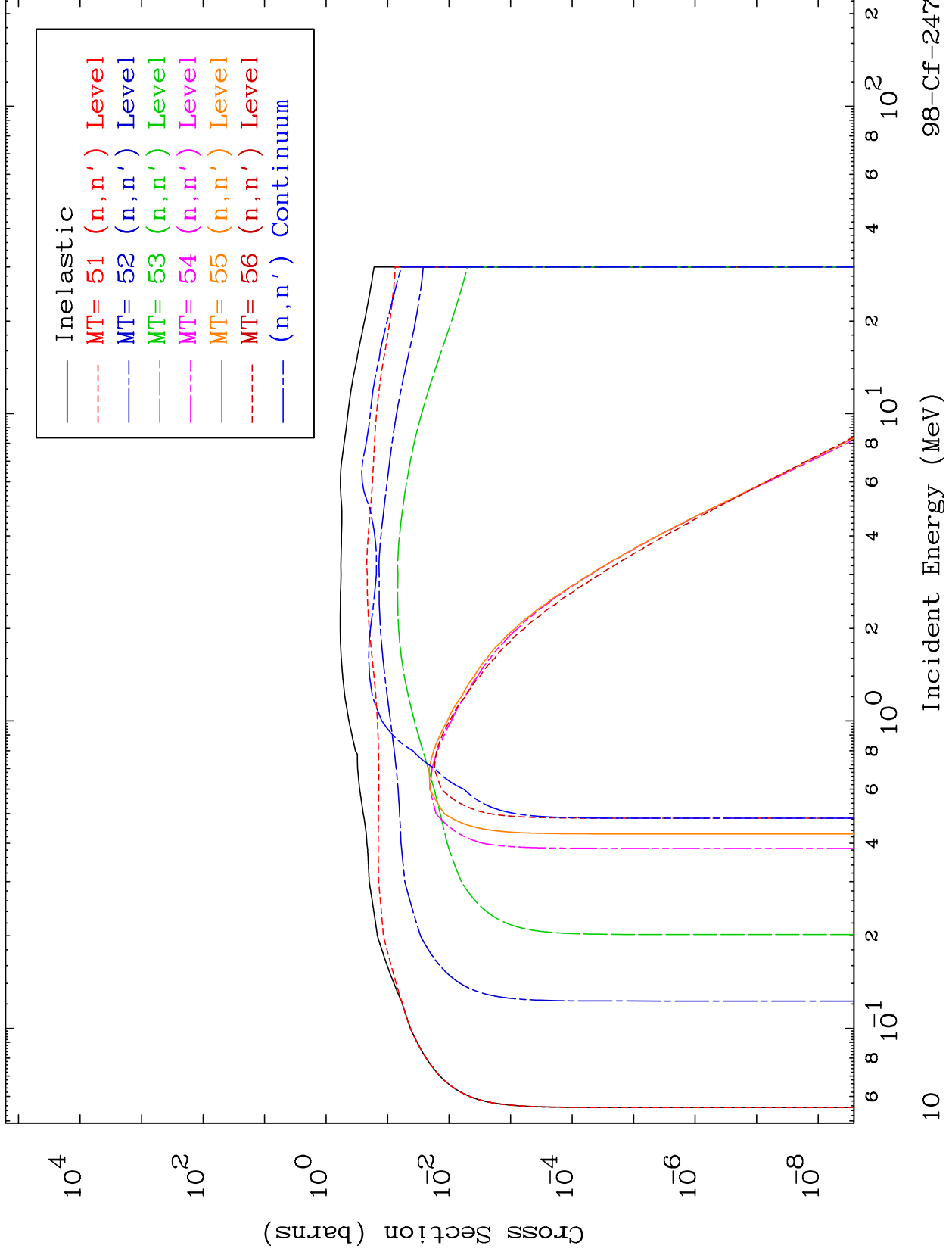
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(n,n') Levels
293 Kelvin Cross Sections

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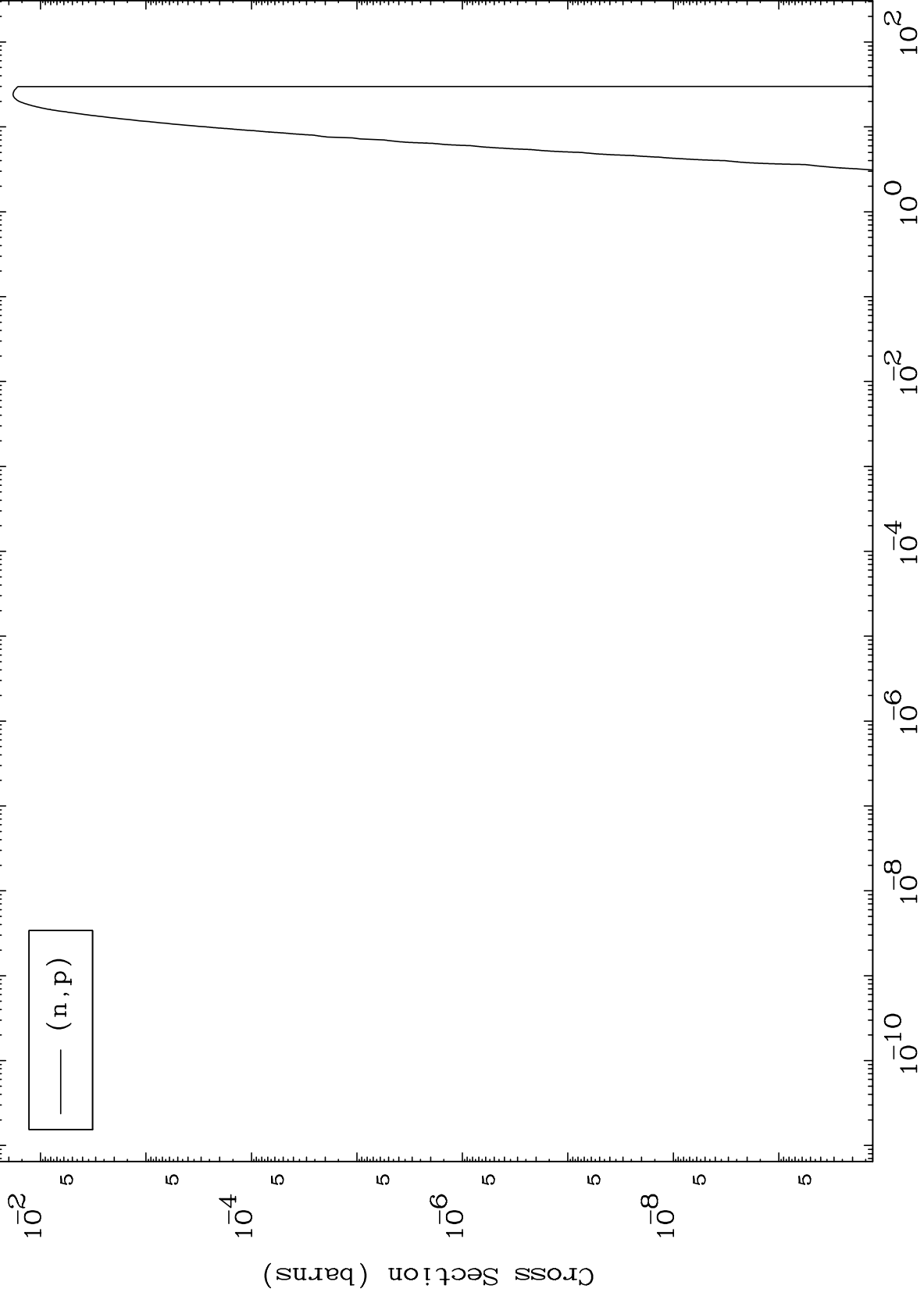
Incident Energy (MeV)

10

MAT 9846

(n,p) Levels
293 Kelvin Cross Sections

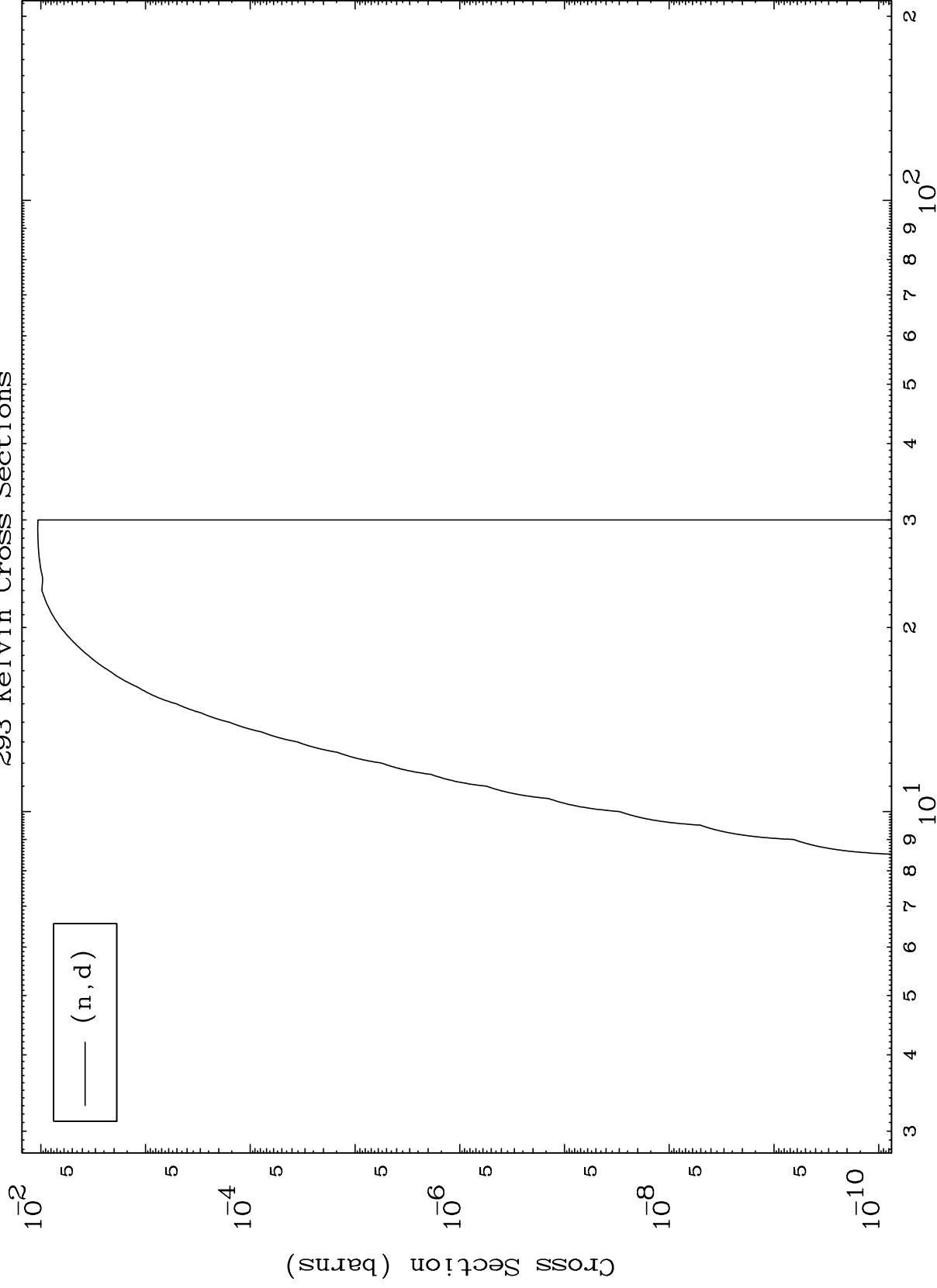
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(n,d) Levels
293 Kelvin Cross Sections

98-Cf-247



12

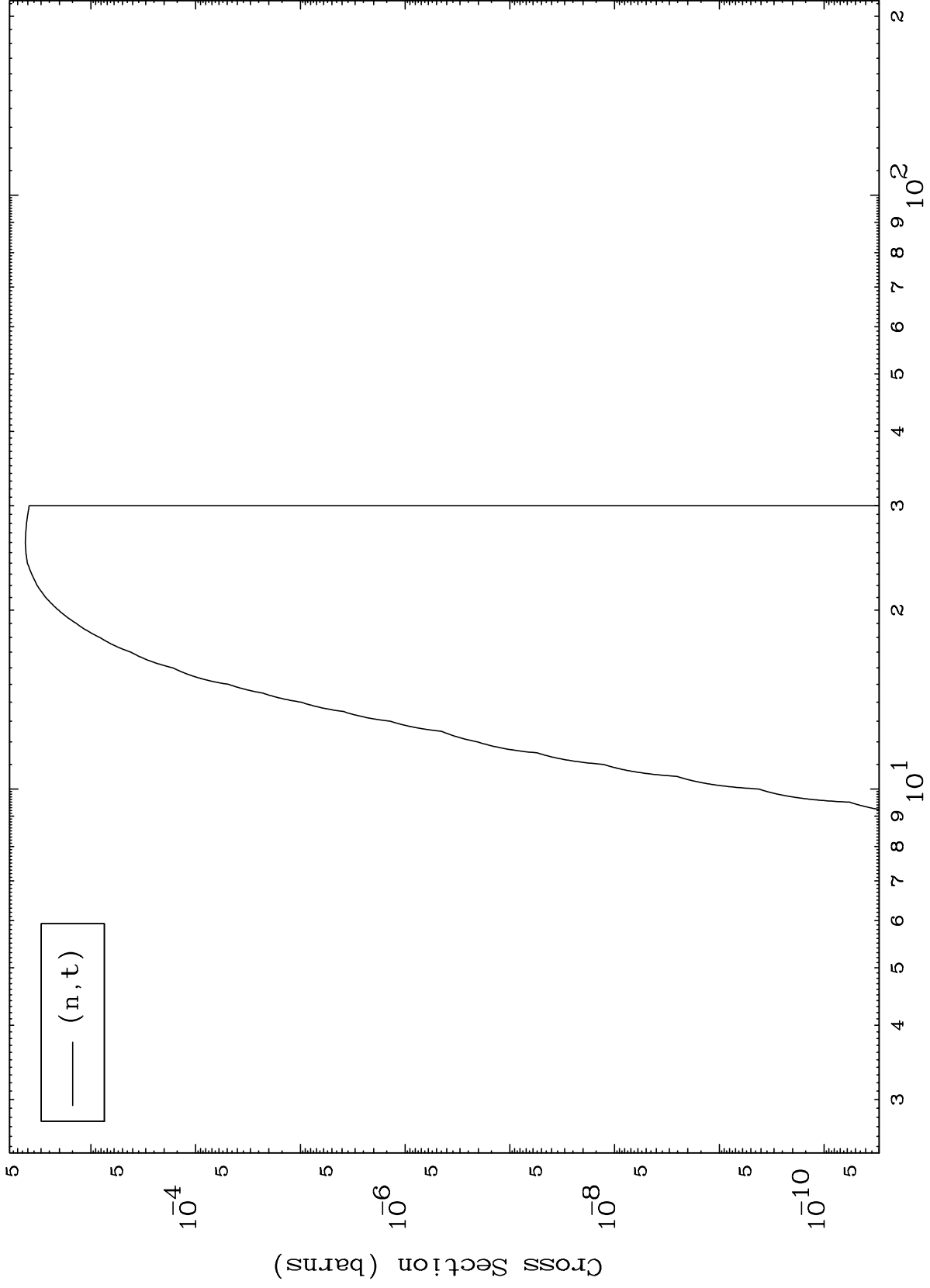
Incident Energy (MeV)

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MAT 9846

(n,t) Levels
293 Kelvin Cross Sections

98-Cf-247



13

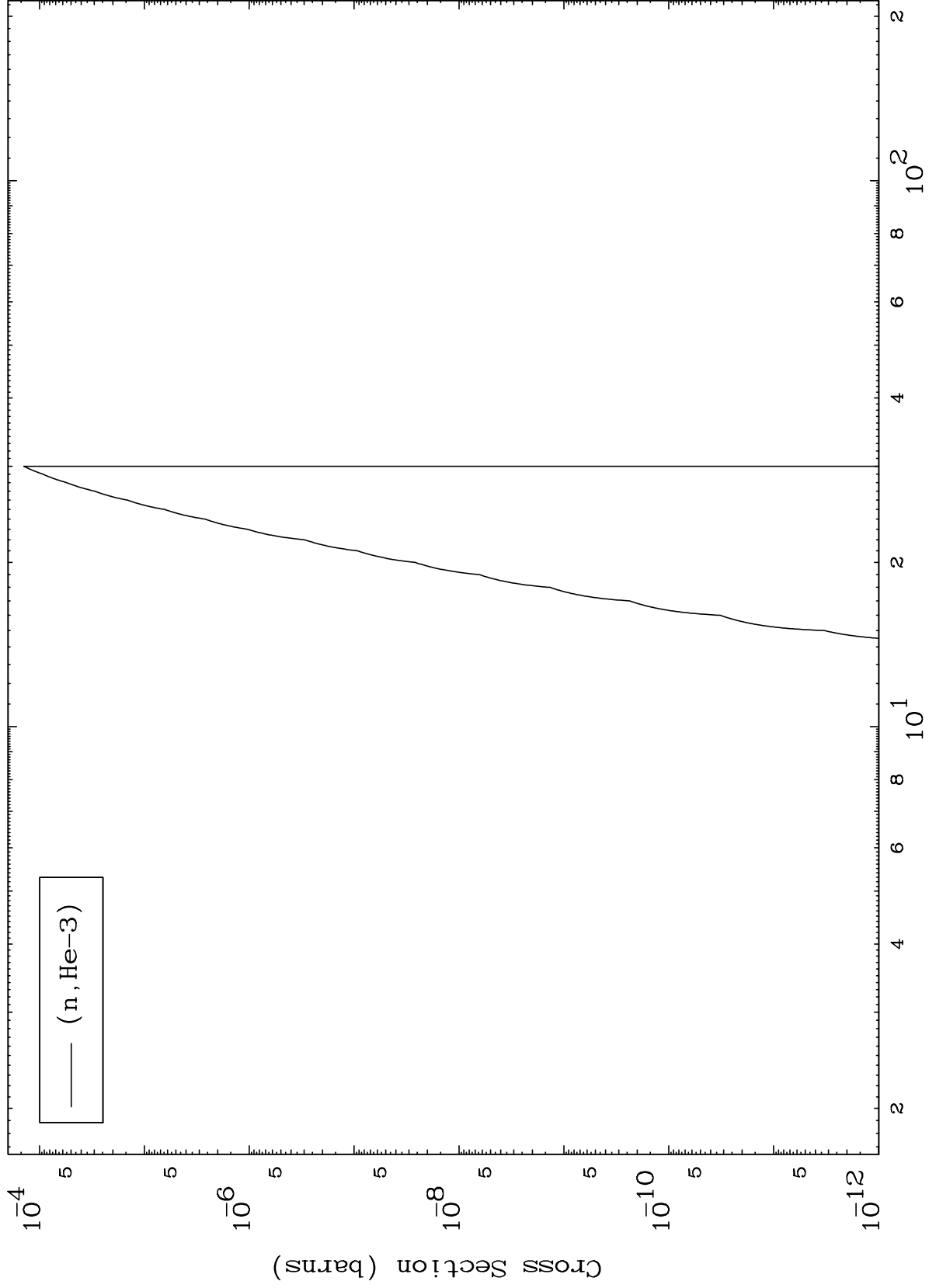
Incident Energy (MeV)

98-Cf-247

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(n,He3) Levels
293 Kelvin Cross Sections

98-Cf-247



14

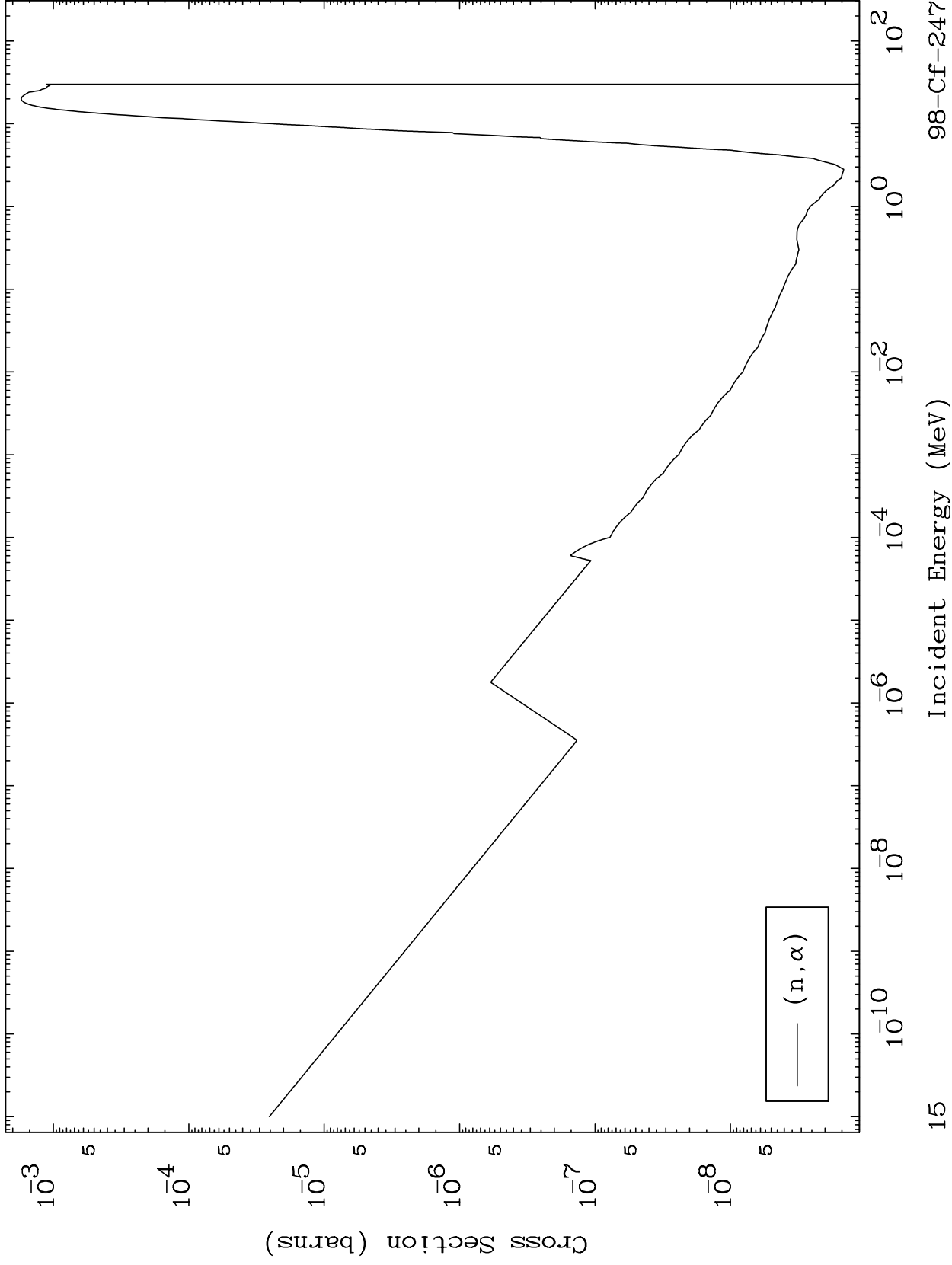
Incident Energy (MeV)

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(n, α) Levels
293 Kelvin Cross Sections

98-Cf-247



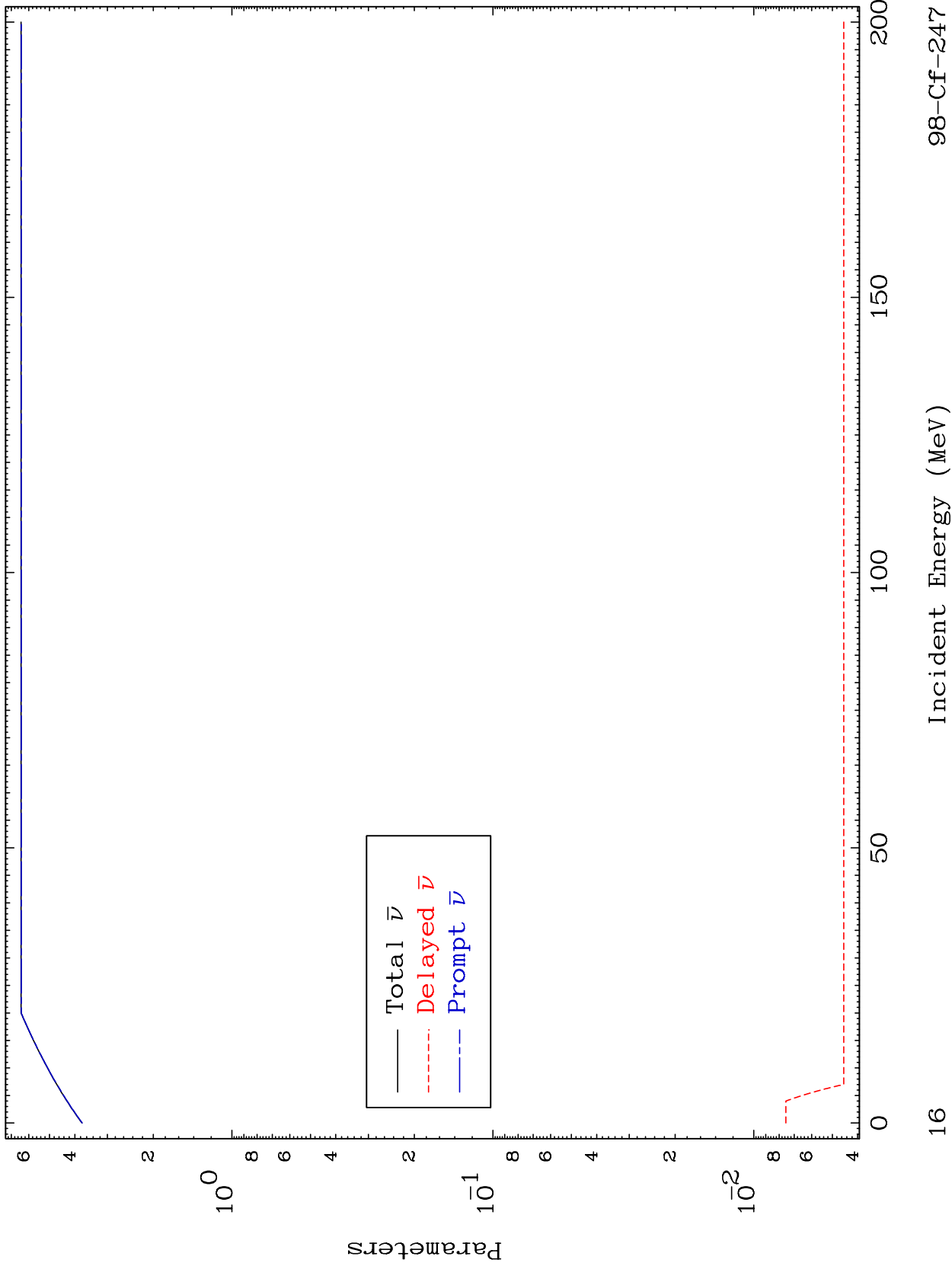
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Energy Release
Parameters

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16

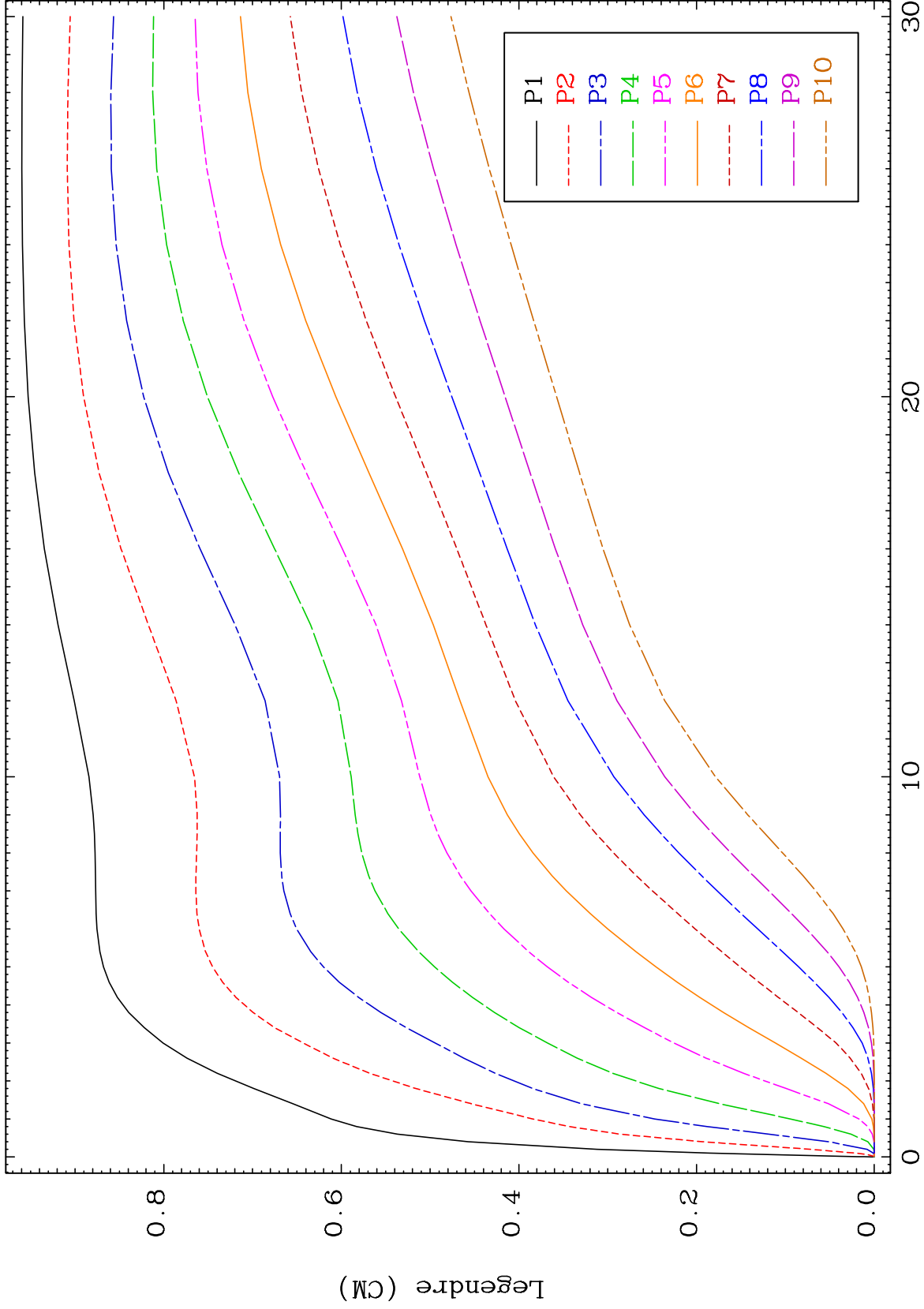
Incident Energy (MeV)

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Elastic Legendre Coefficients

98-Cf-247



17

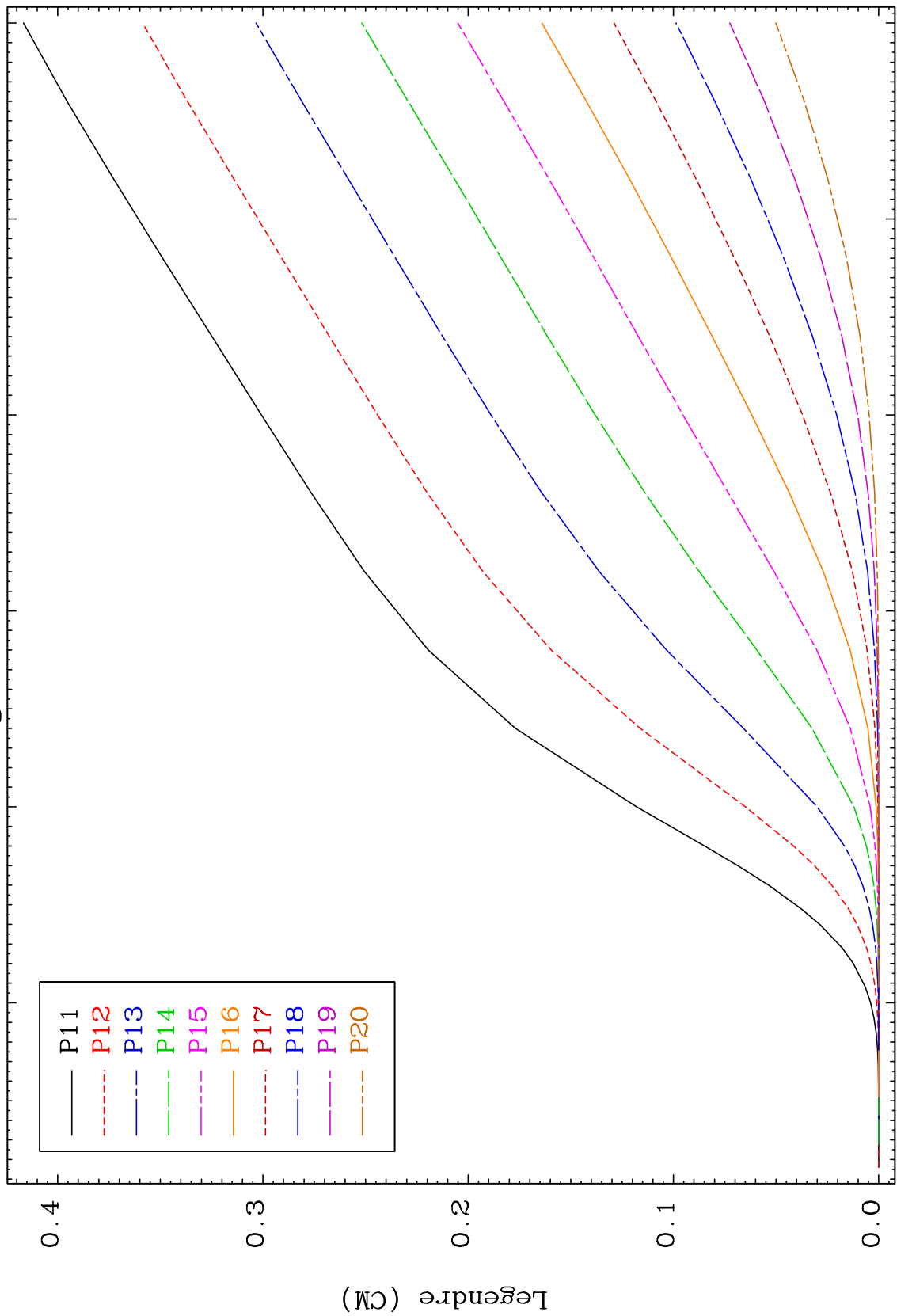
Incident Energy (MeV)

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MAT 9846

Elastic Legendre Coefficients

98-Cf-247



18

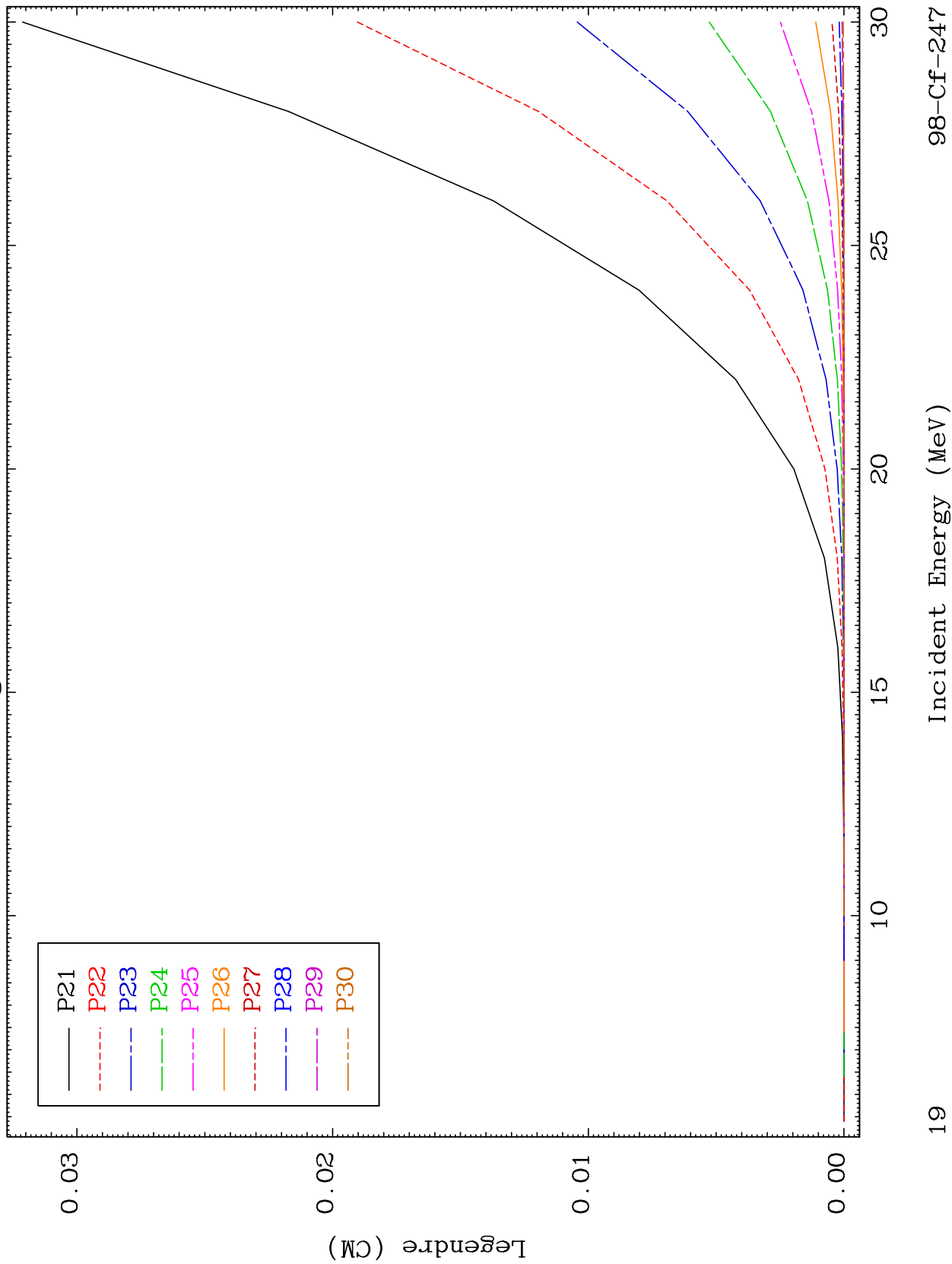
Incident Energy (MeV)

98-Cf-247

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Elastic Legendre Coefficients

98-Cf-247



19

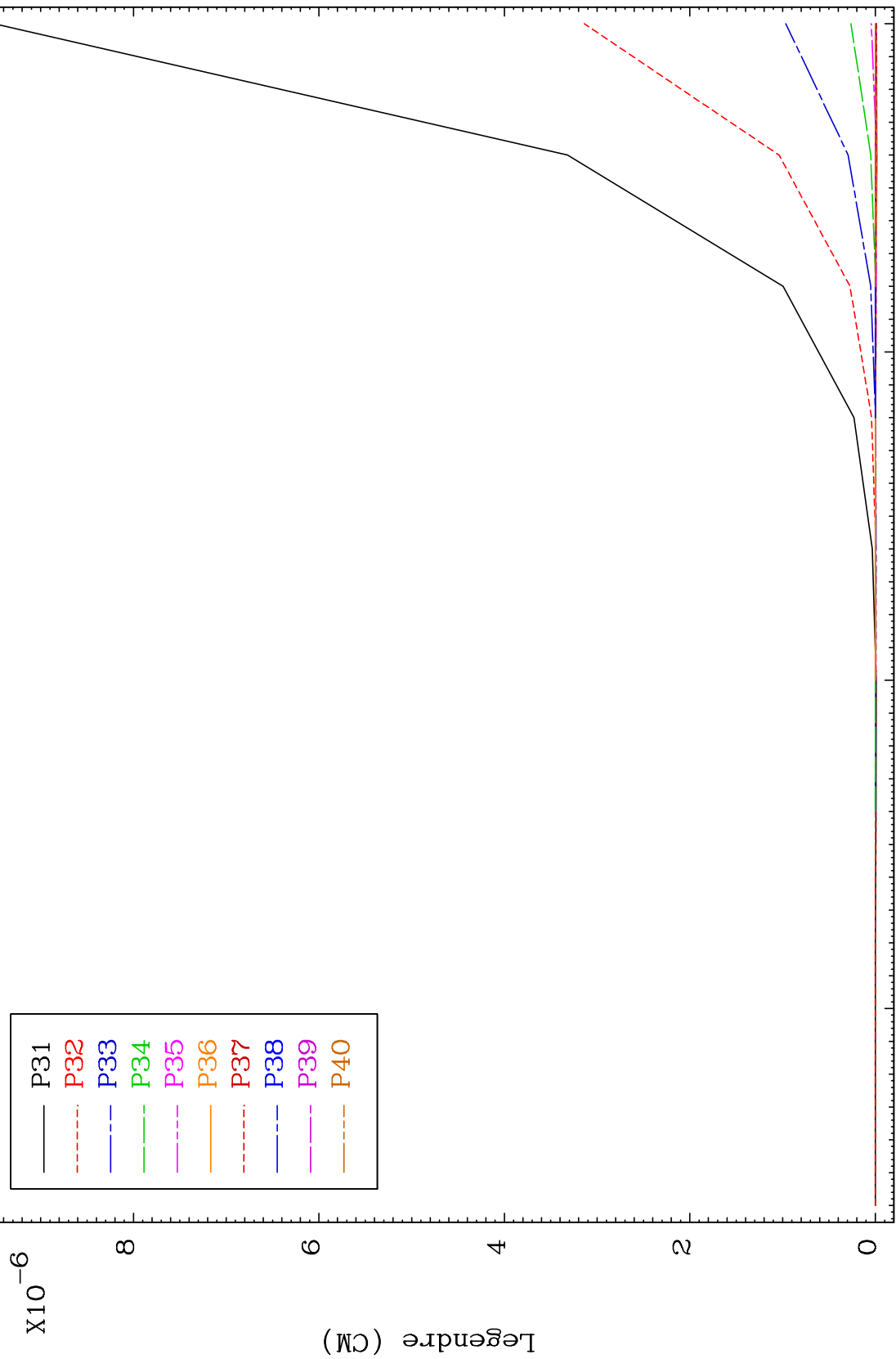
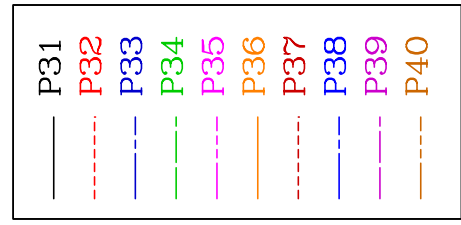
Incident Energy (MeV)

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Elastic Legendre Coefficients

98-Cf-247



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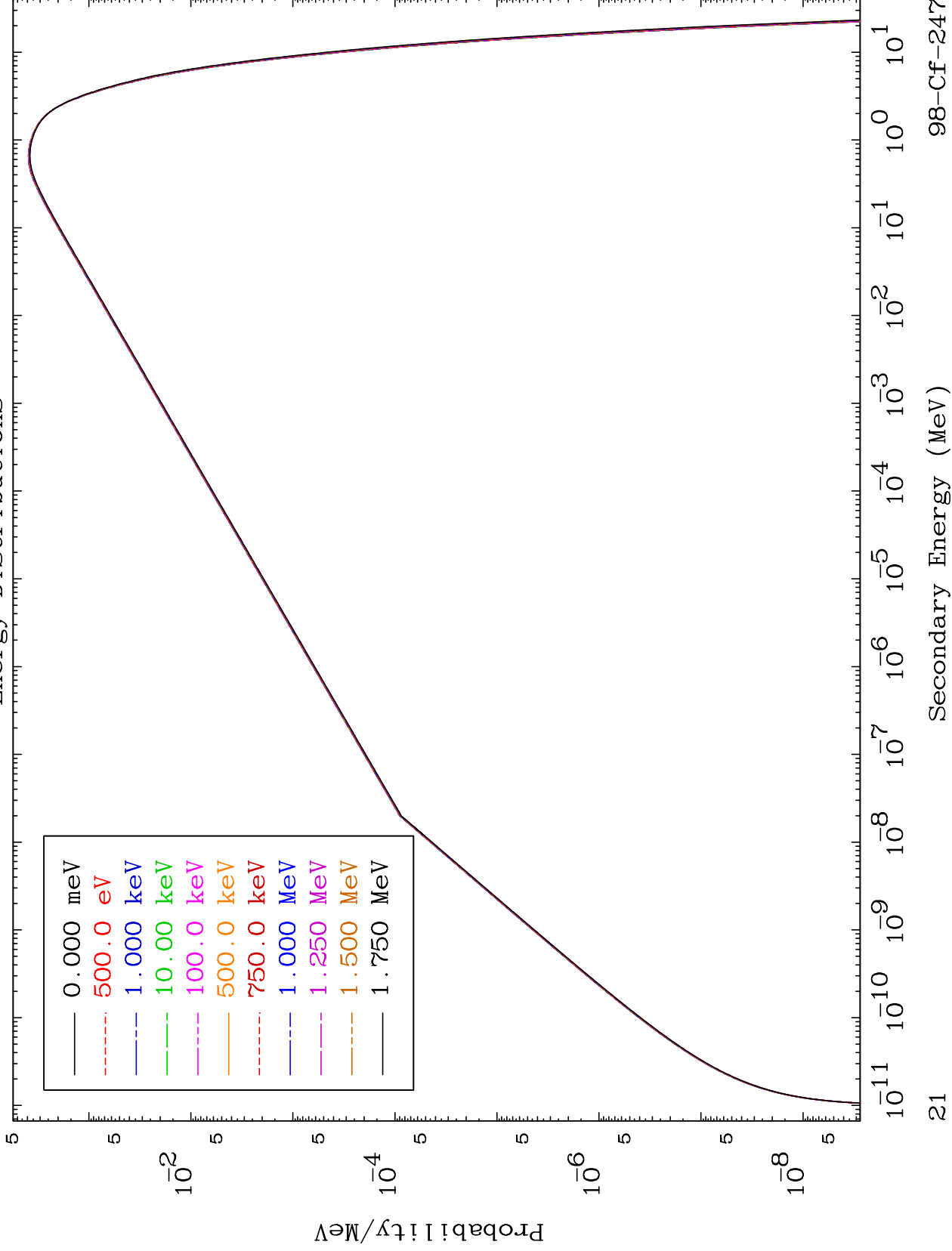
Incident Energy (MeV)

20

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Fission Energy Distributions

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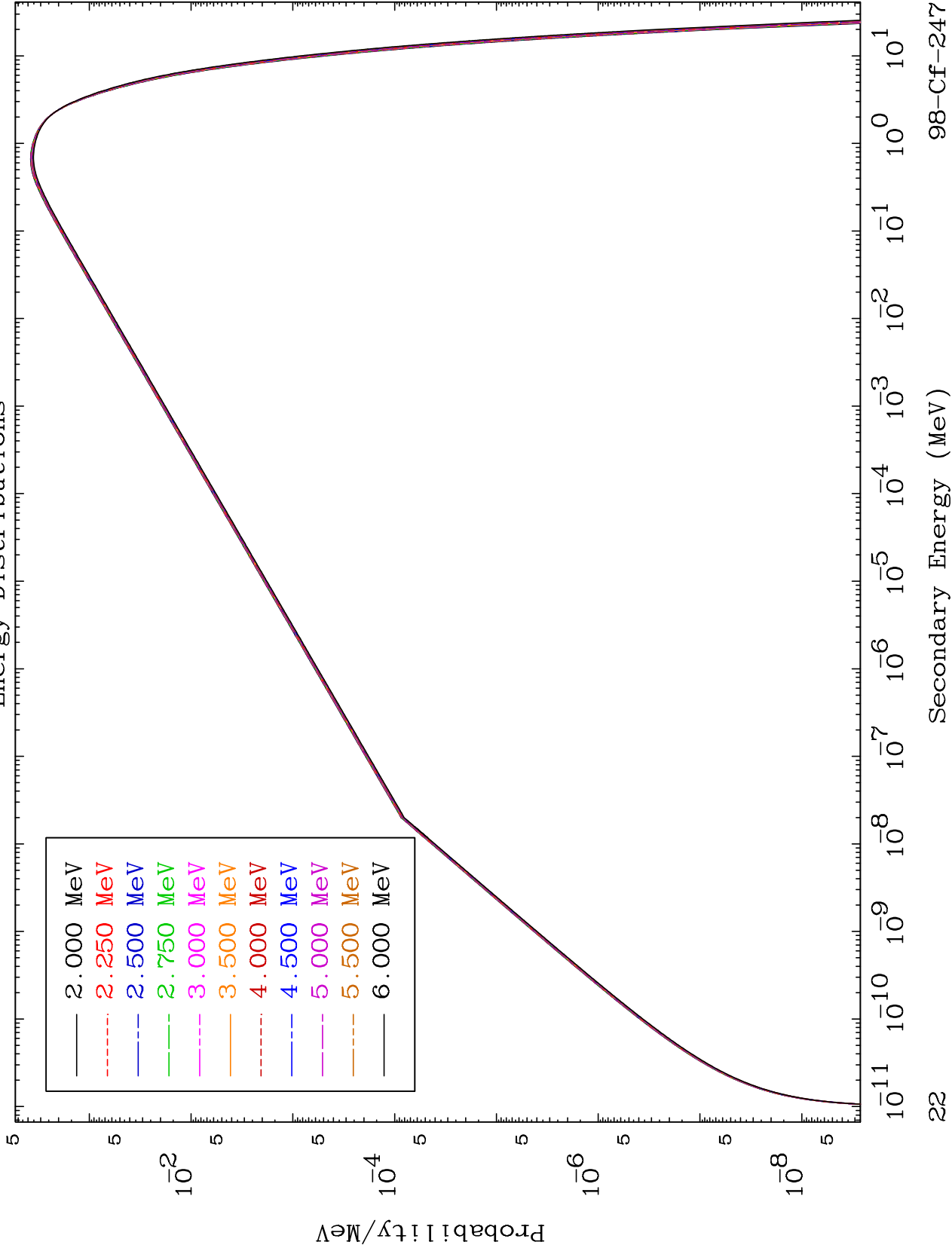
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Secondary Energy (MeV)

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Fission Energy Distributions

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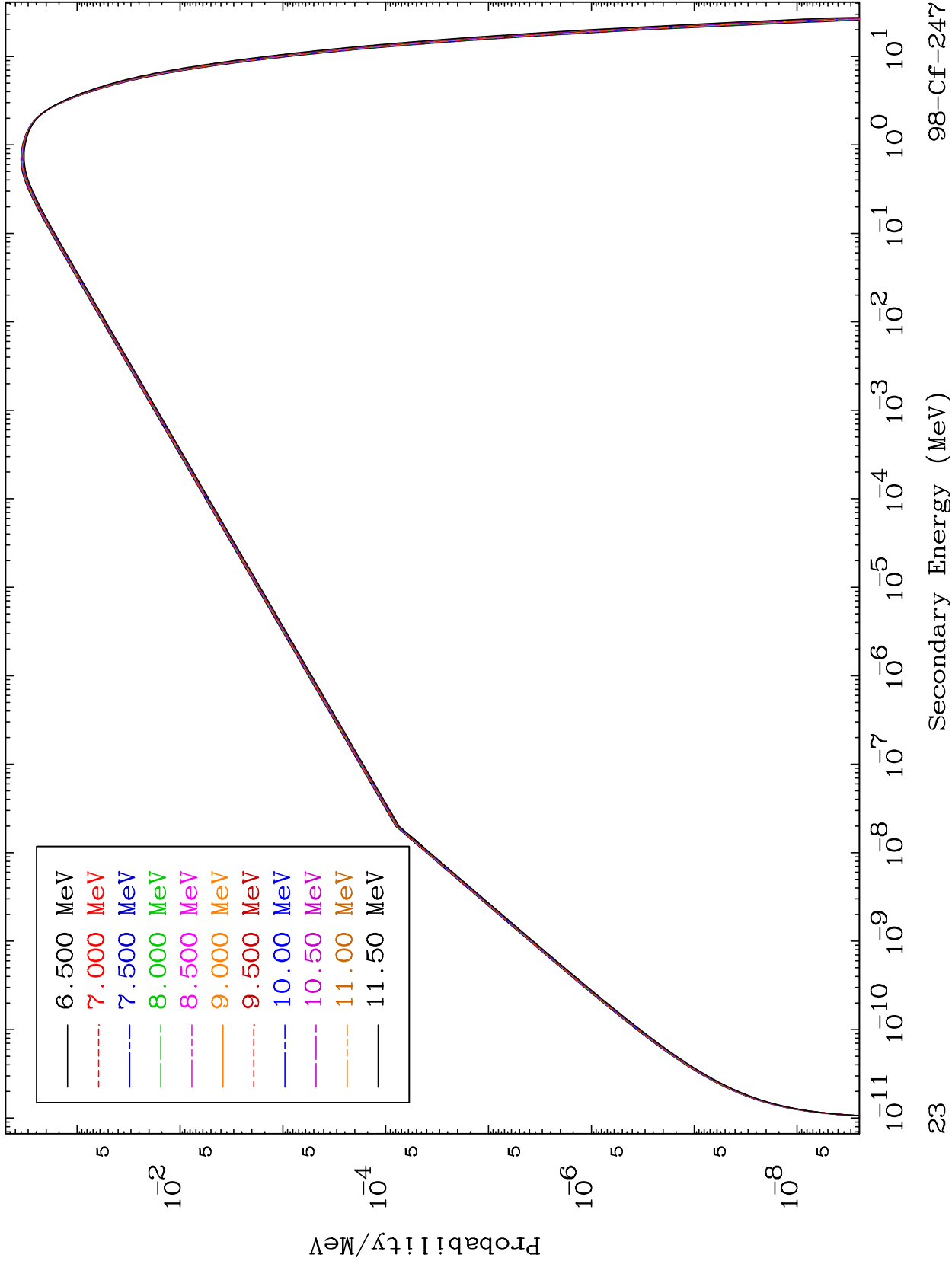
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Secondary Energy (MeV)

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Fission Energy Distributions

98-Cf-247



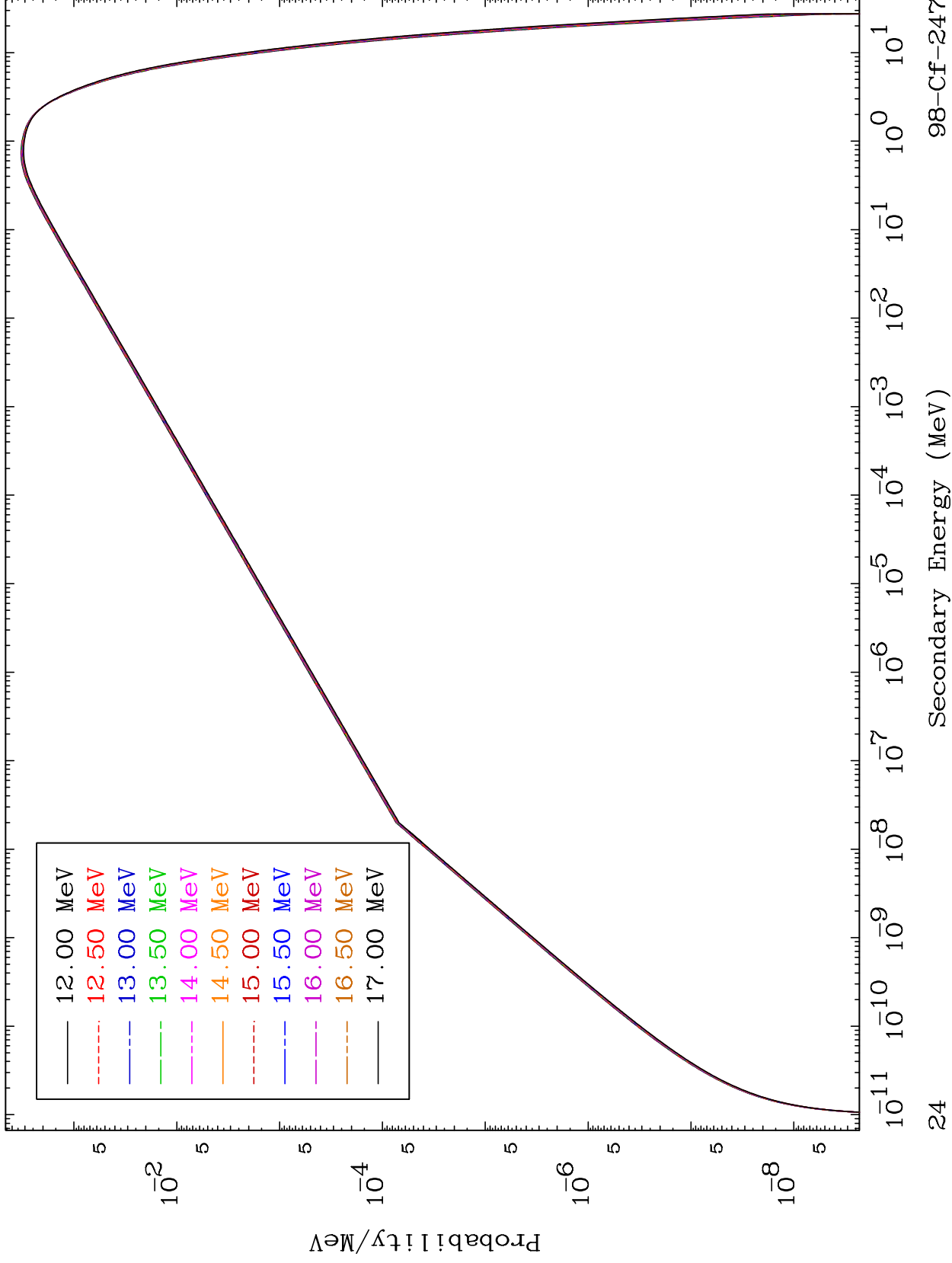
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Secondary Energy (MeV)

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Fission Energy Distributions

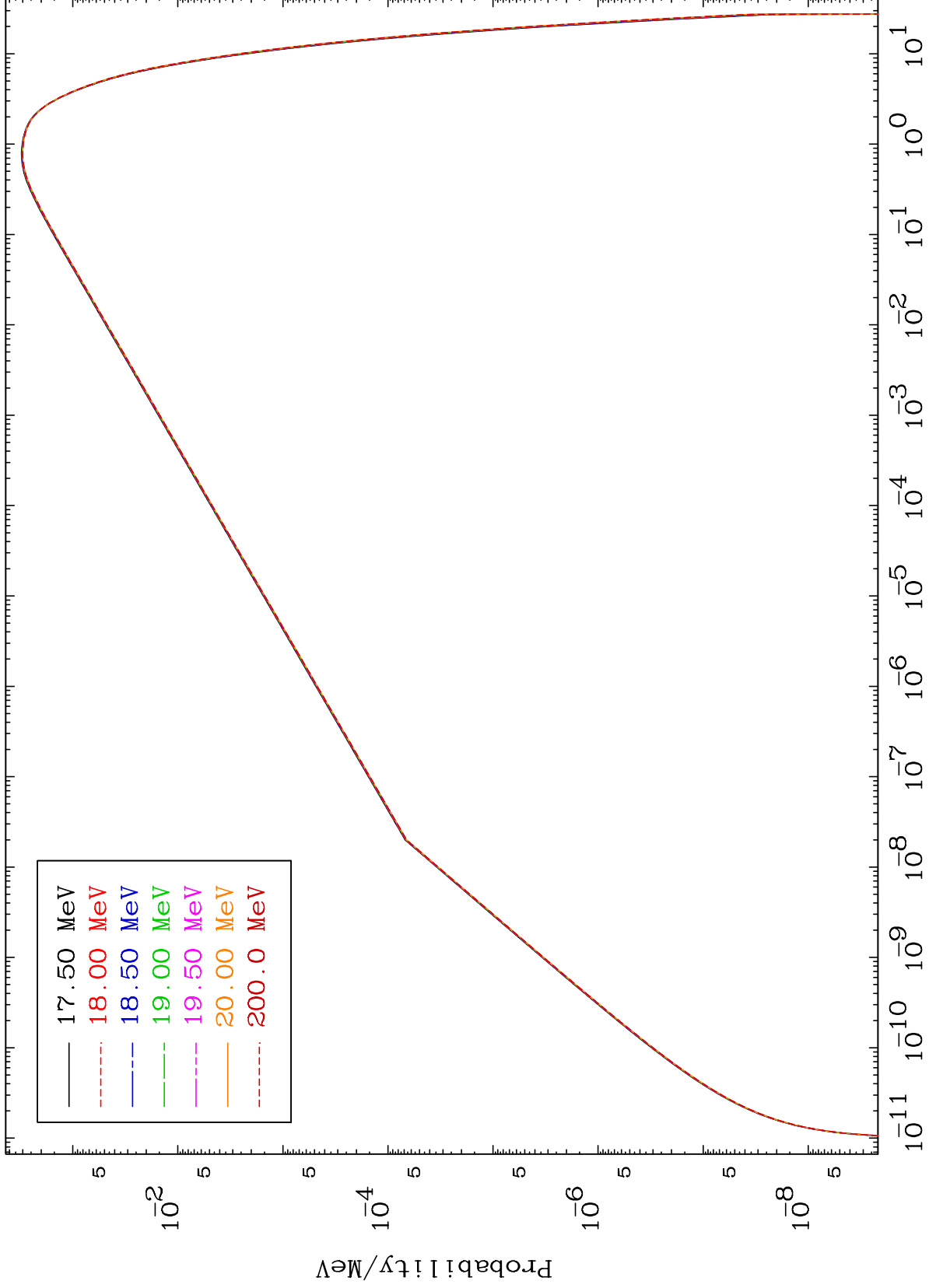
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Fission Energy Distributions

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25

Secondary Energy (MeV)

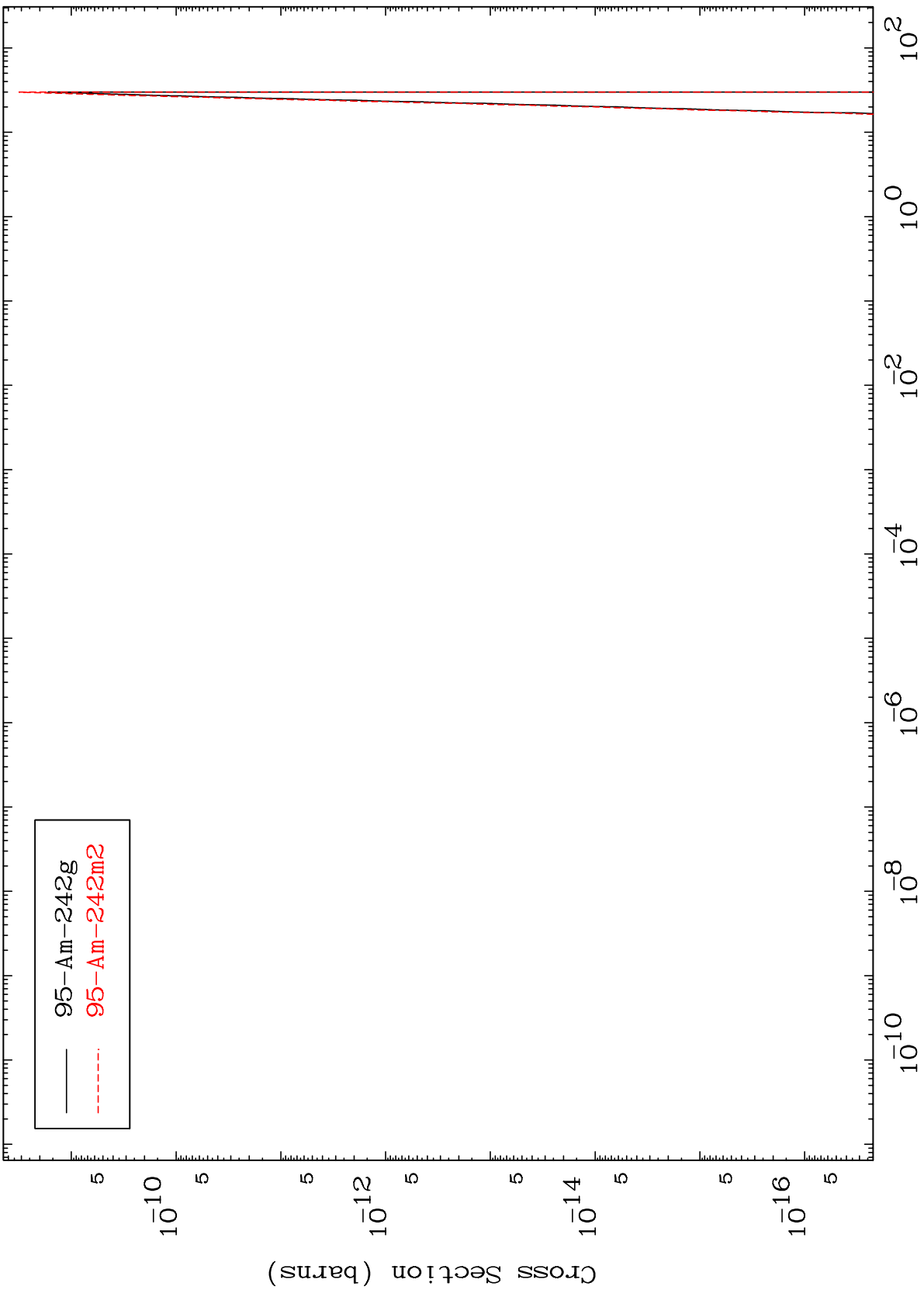
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(n,n') p α

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Radionuclide Production Cross Section



26

Incident Energy (MeV)

98-Cf-247