

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

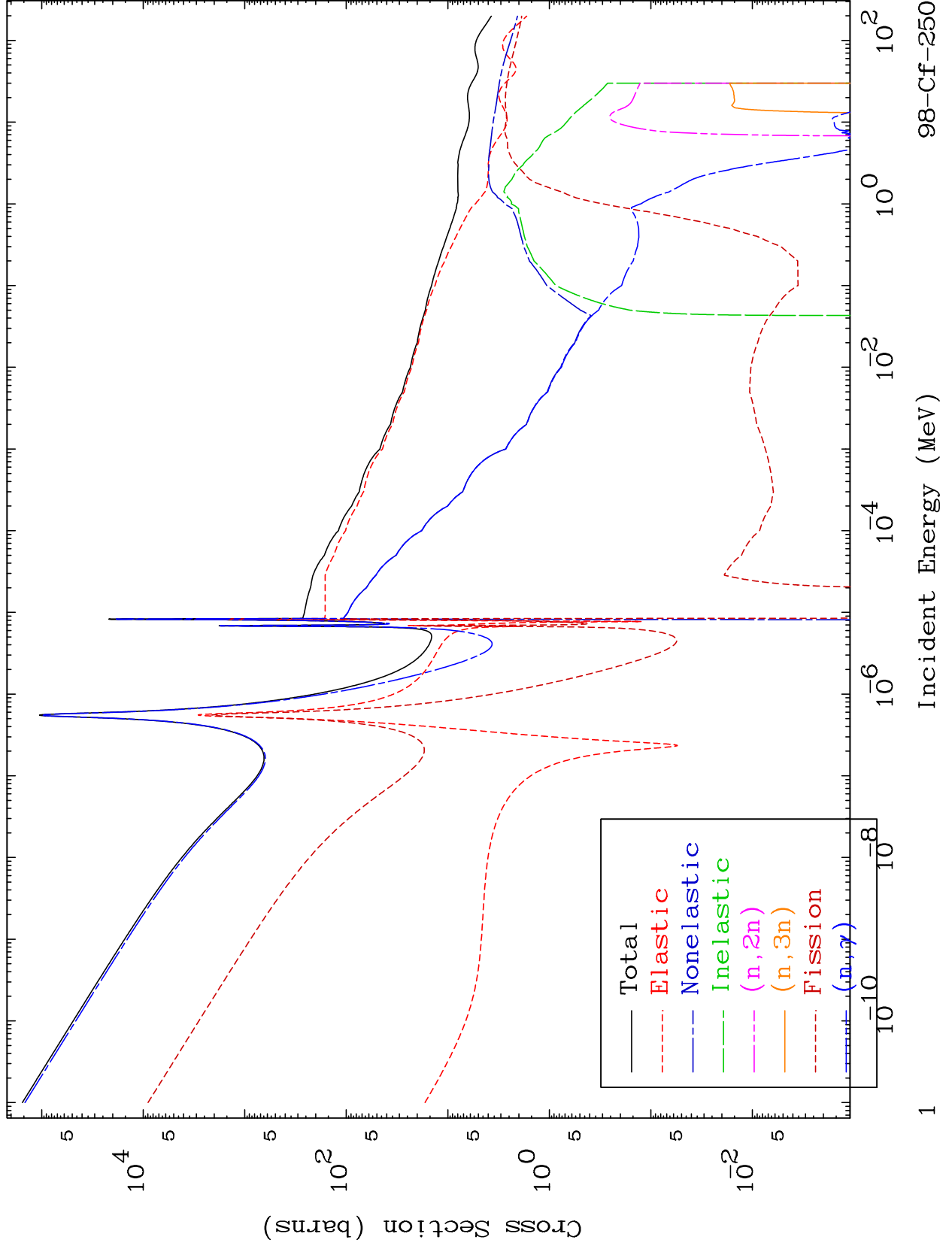
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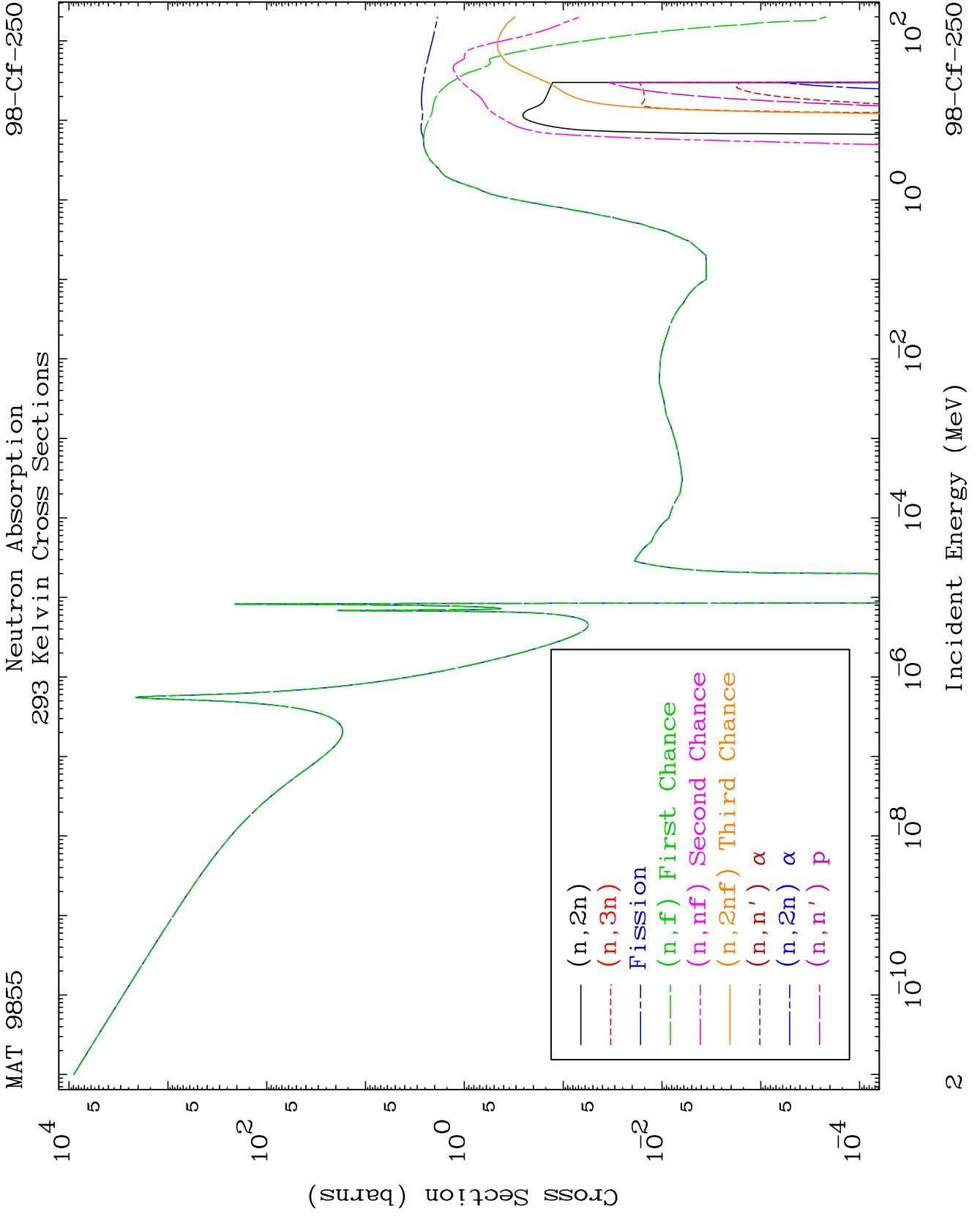
Press Mouse Button to Start

MAT 9855

Neutron Major
293 Kelvin Cross Sections

98-Cf-250

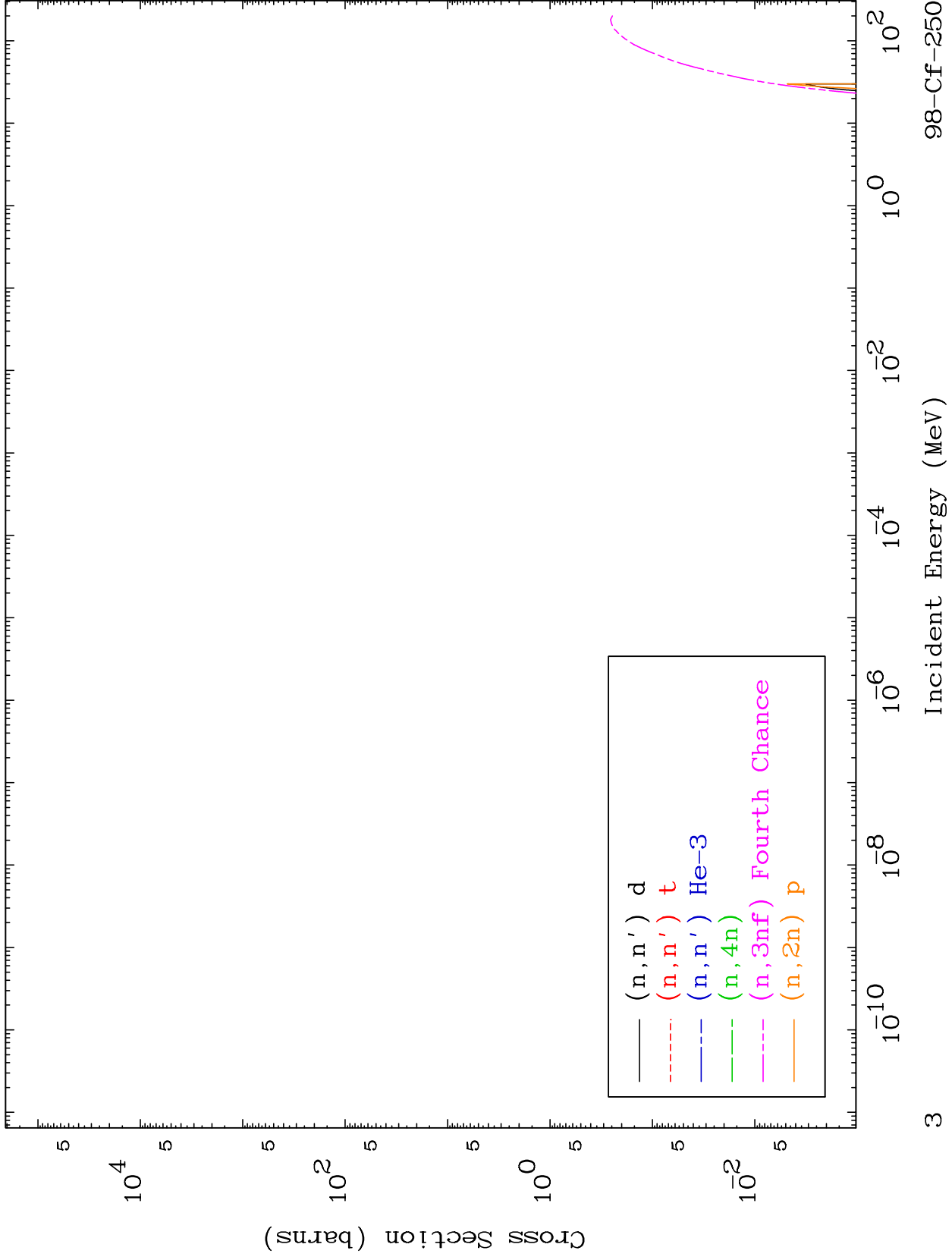




MAT 9855

Neutron Absorption
293 Kelvin Cross Sections

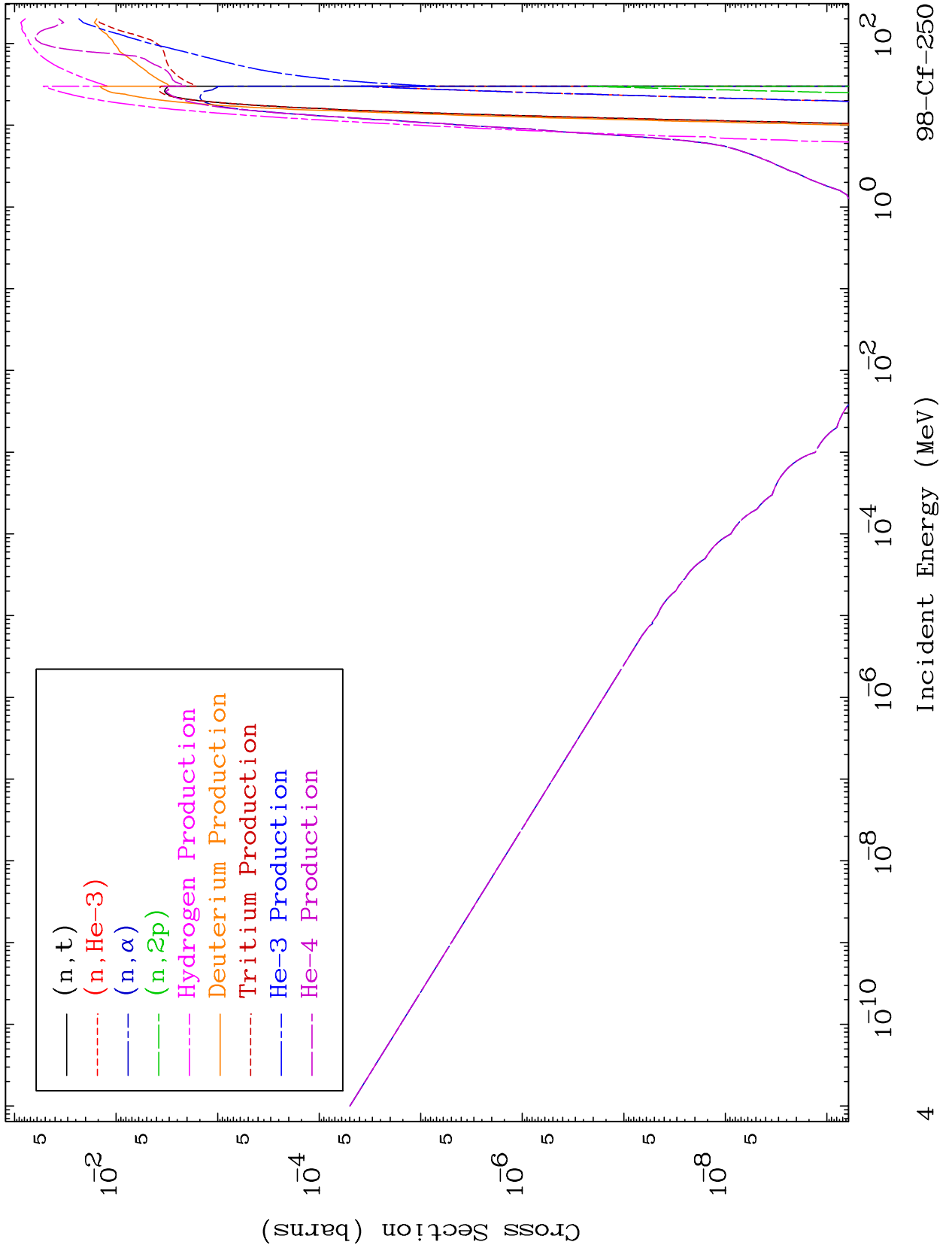
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MAT 9855

Neutron Absorption
293 Kelvin Cross Sections

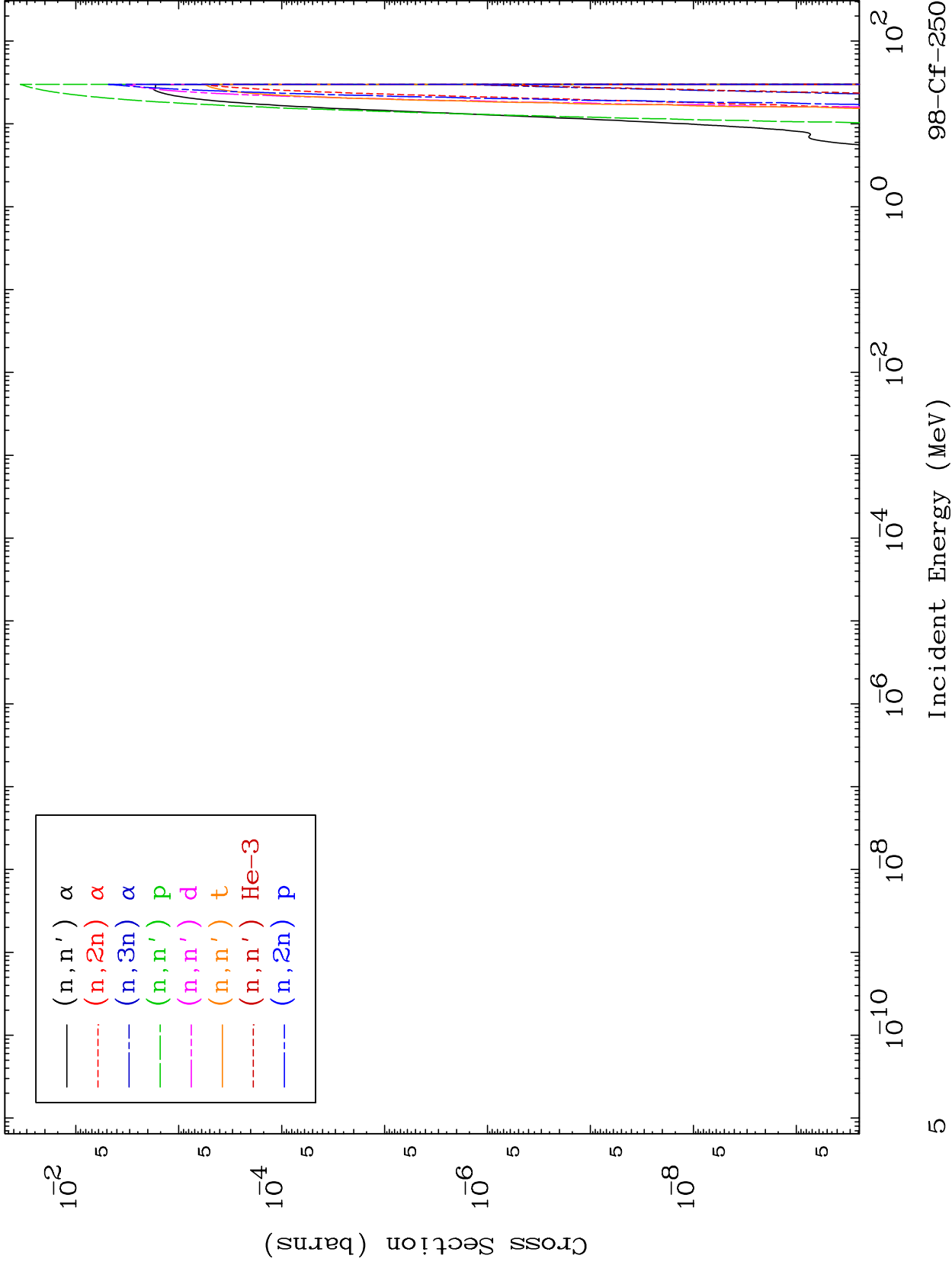
98-Cf-250



MAT 9855

Charged Particle
293 Kelvin Cross Sections

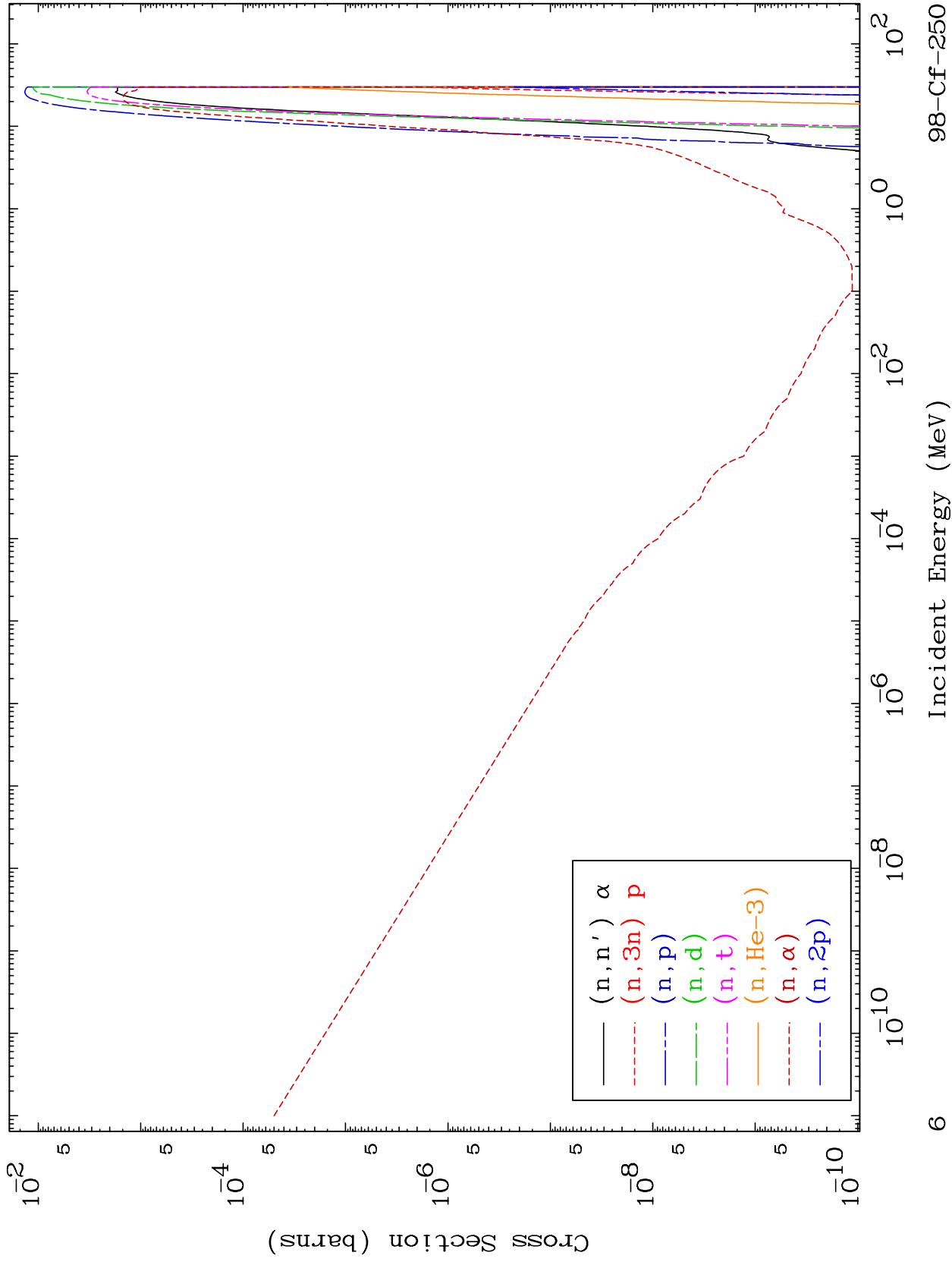
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MAT 9855

Charged Particle
293 Kelvin Cross Sections

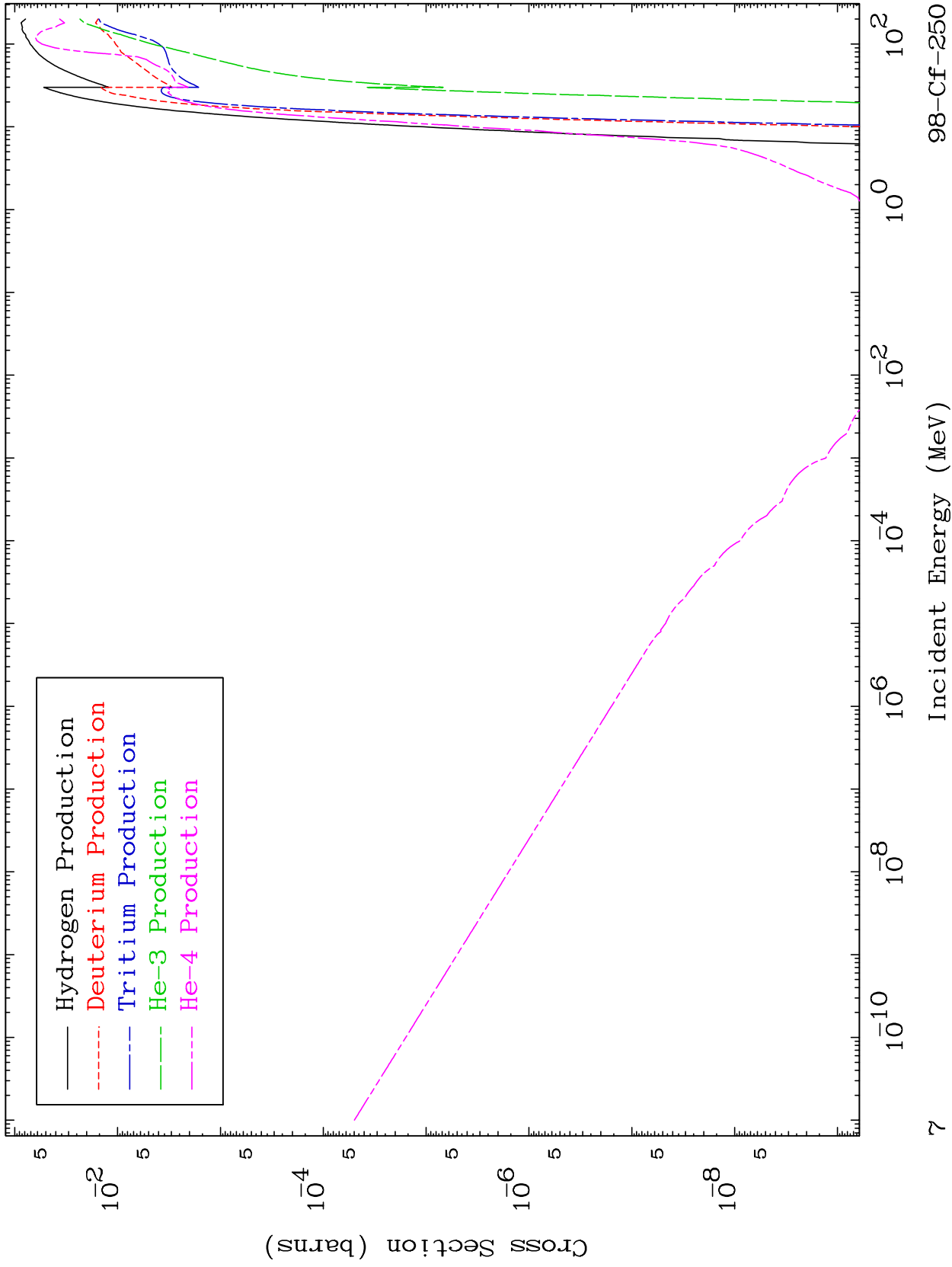
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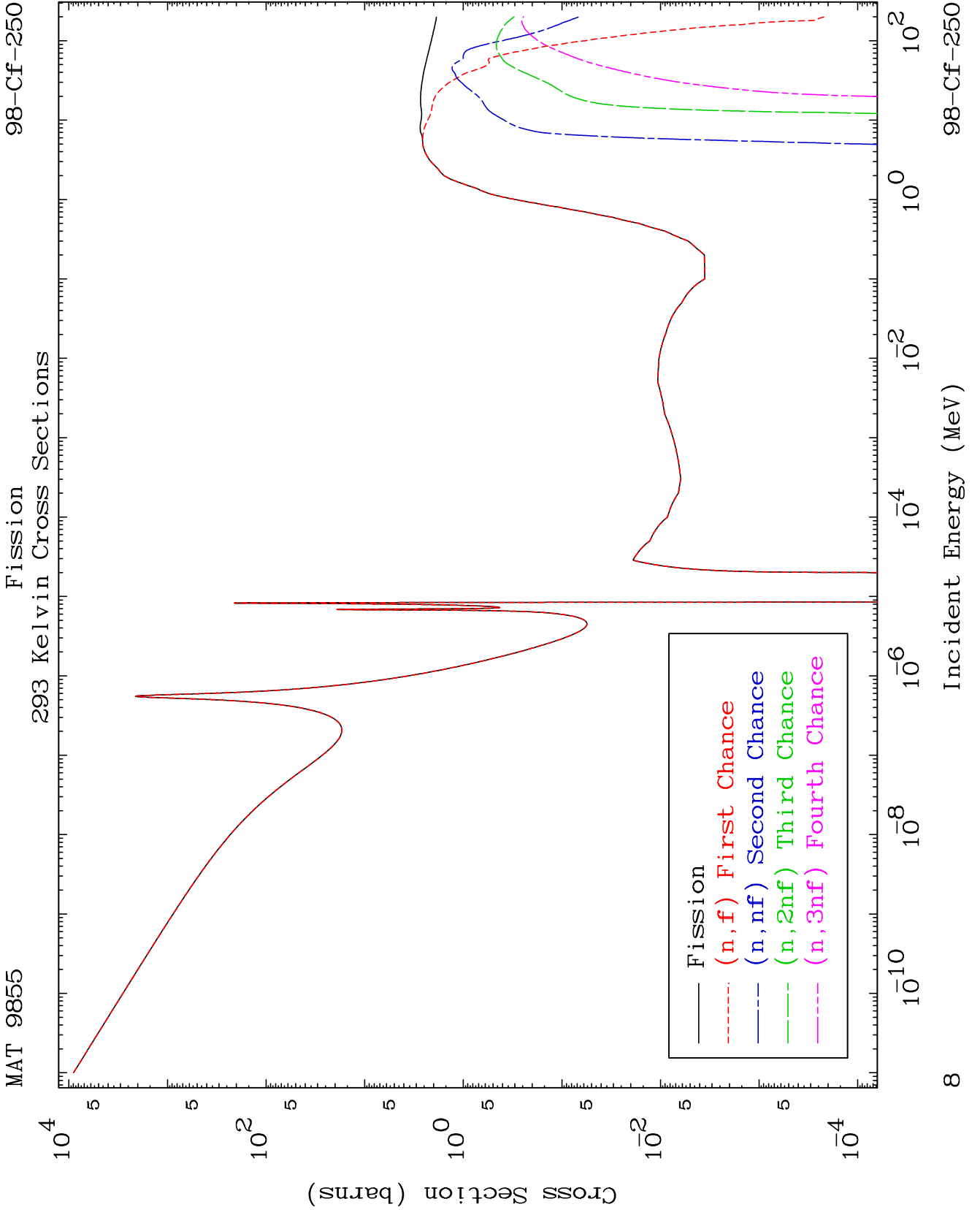


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Particle Production
293 Kelvin Cross Sections

98-Cf-250



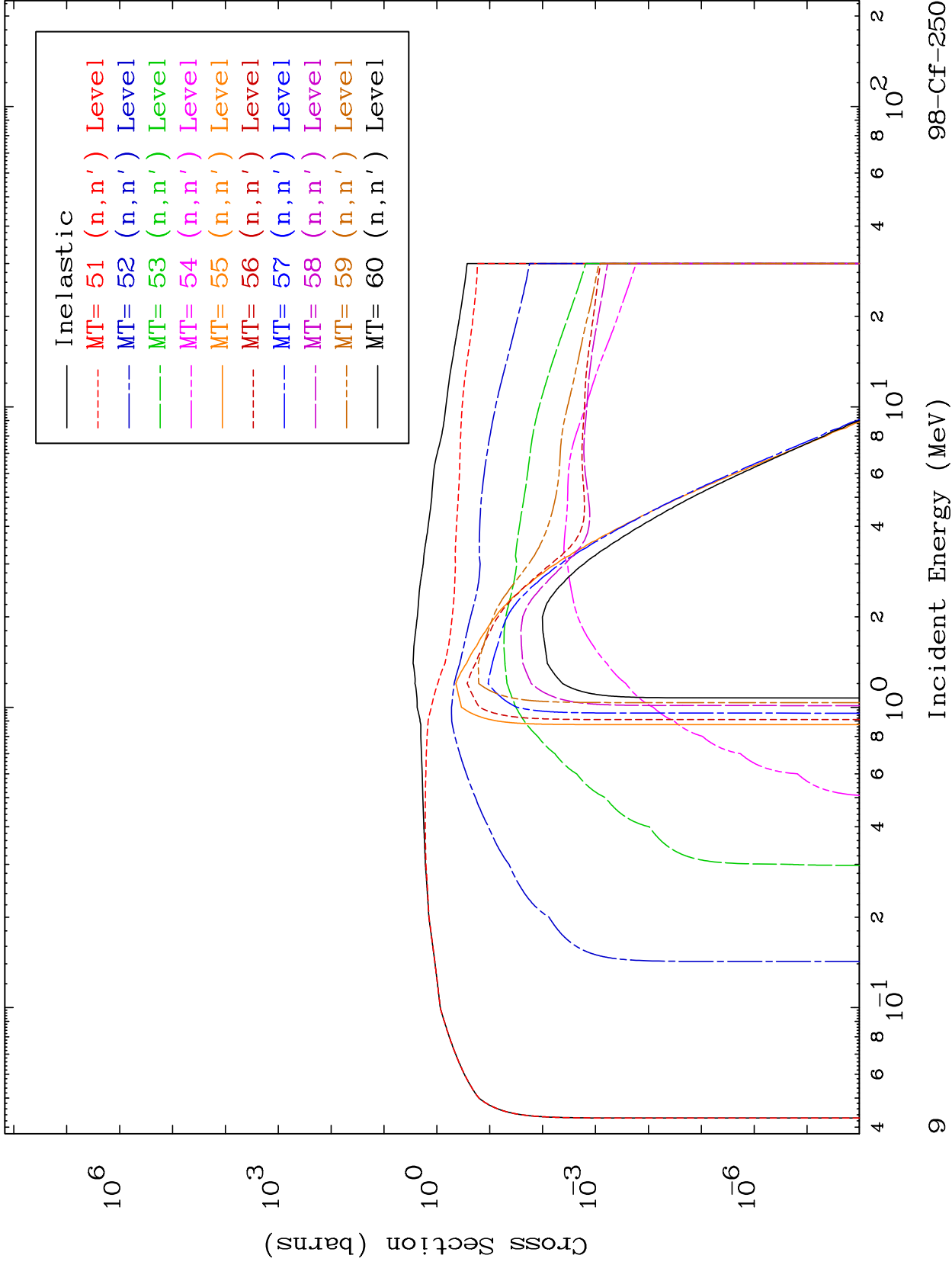


MAT 9855

(n,n') Levels

98-Cf-250

293 Kelvin Cross Sections

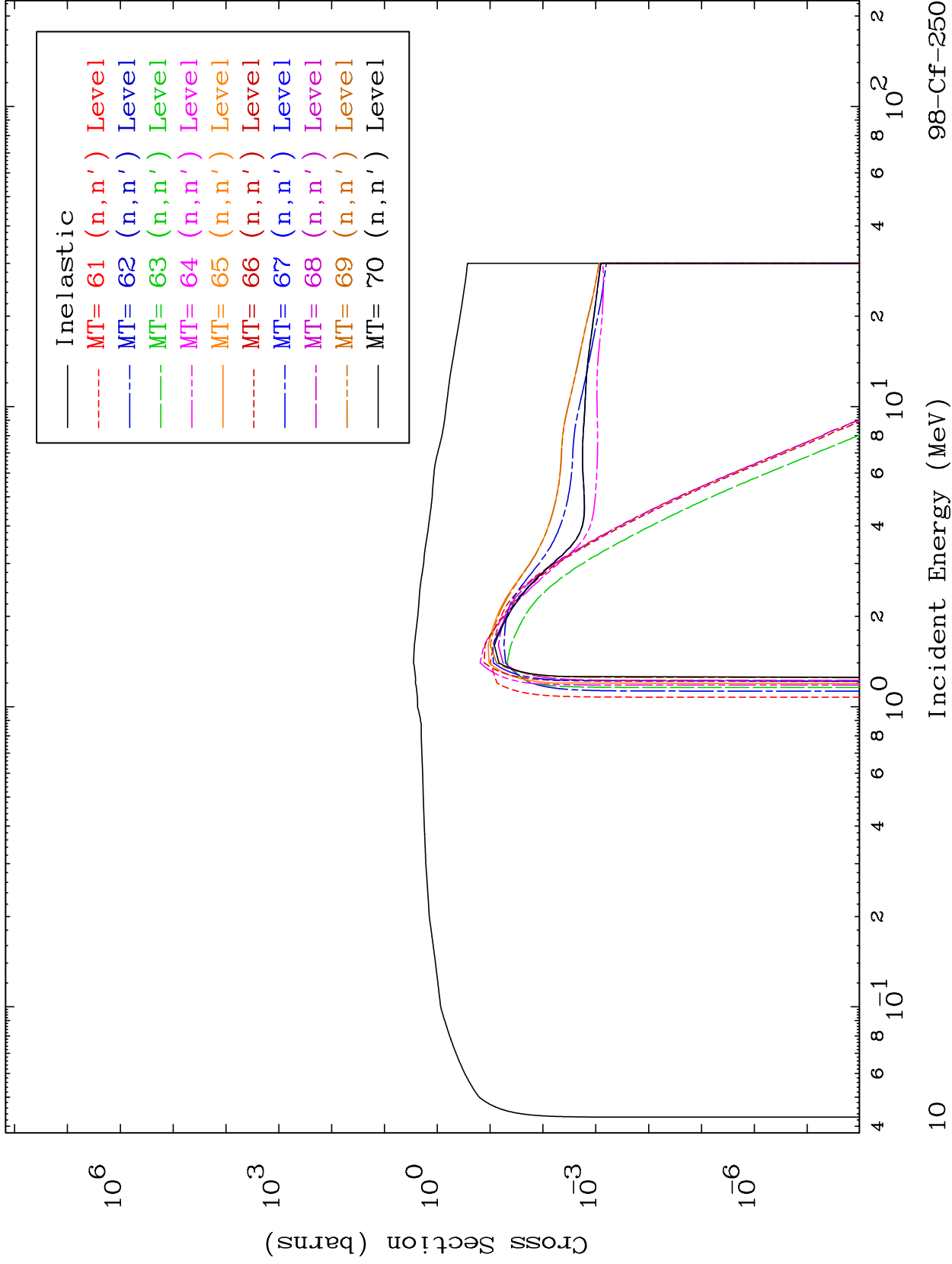


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(n,n') Levels

98-Cf-250

293 Kelvin Cross Sections



10

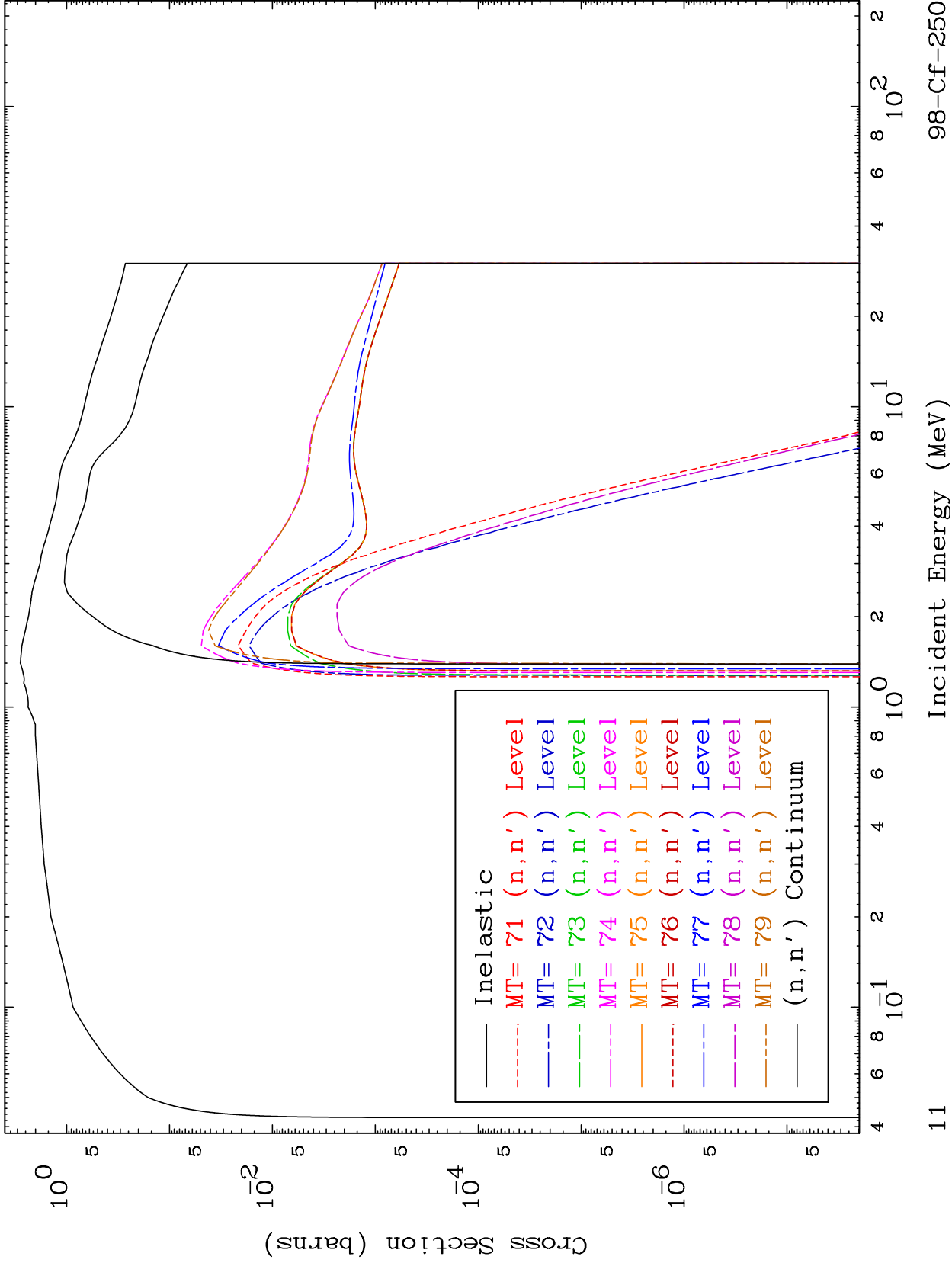
Incident Energy (MeV)

98-Cf-250

MAT 9855

(n,n') Levels
293 Kelvin Cross Sections

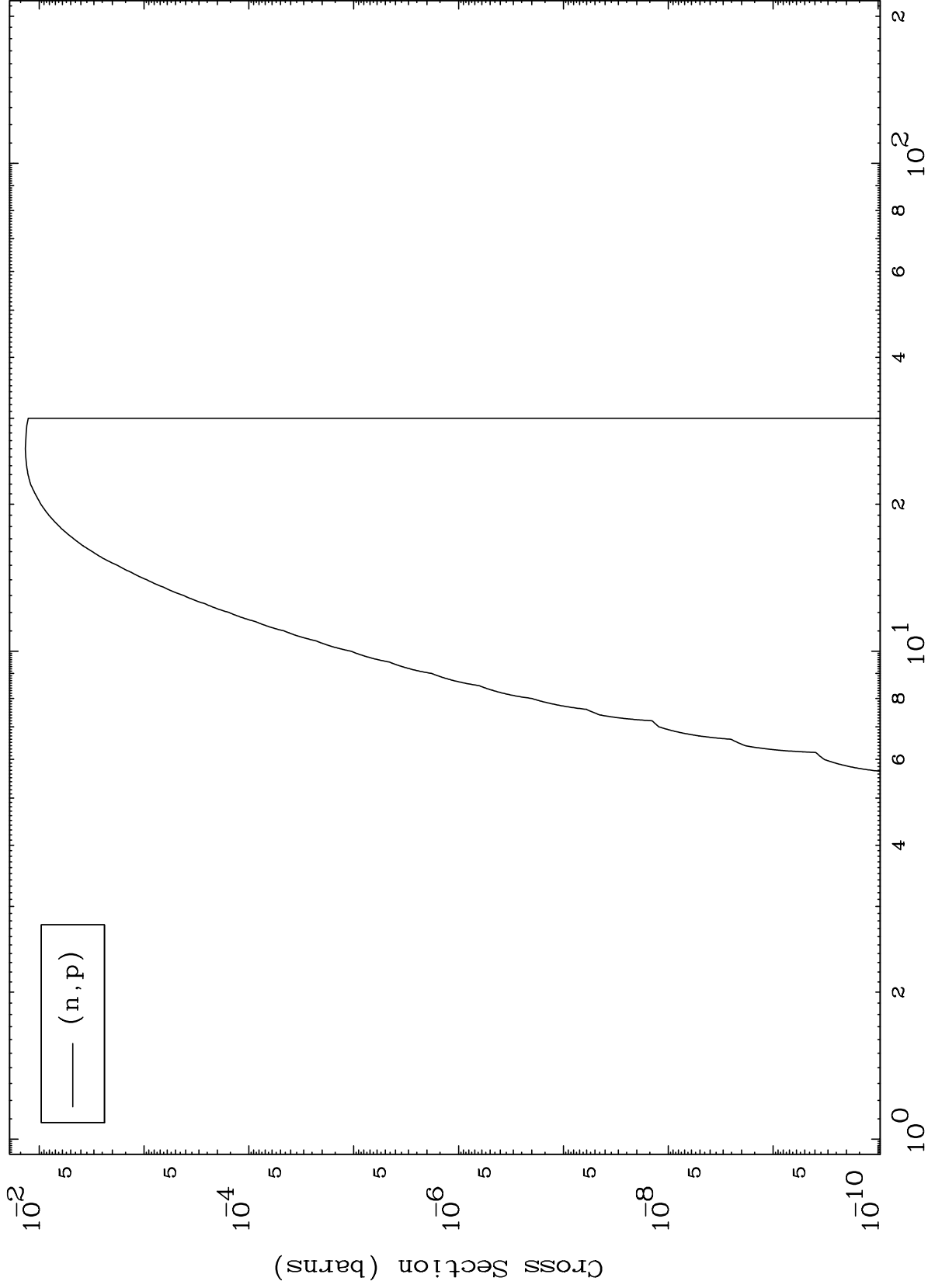
98-Cf-250



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(n,p) Levels
293 Kelvin Cross Sections

98-Cf-250



12

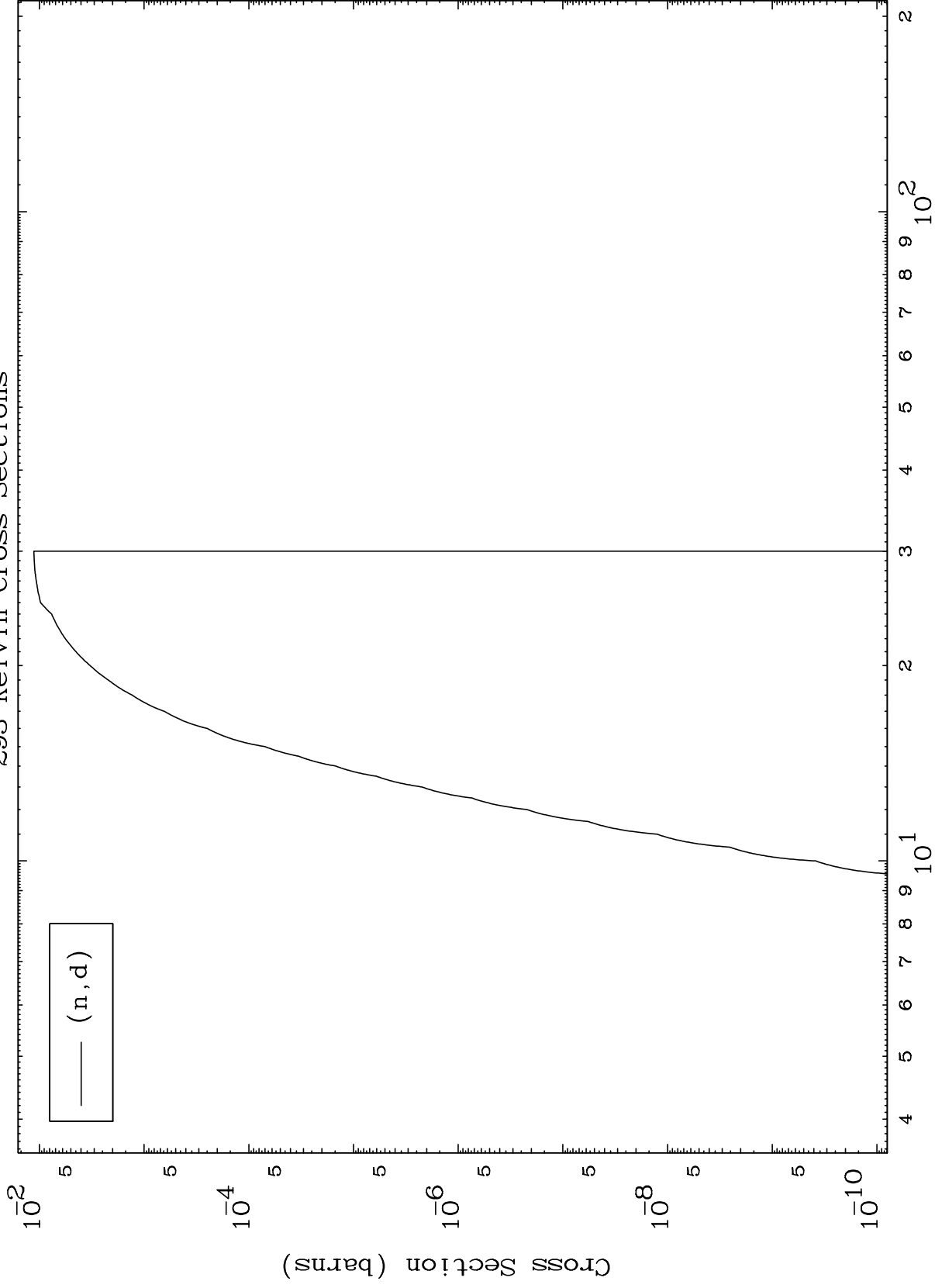
Incident Energy (MeV)

98-Cf-250

MAT 9855

(n,d) Levels
293 Kelvin Cross Sections

98-Cf-250



13

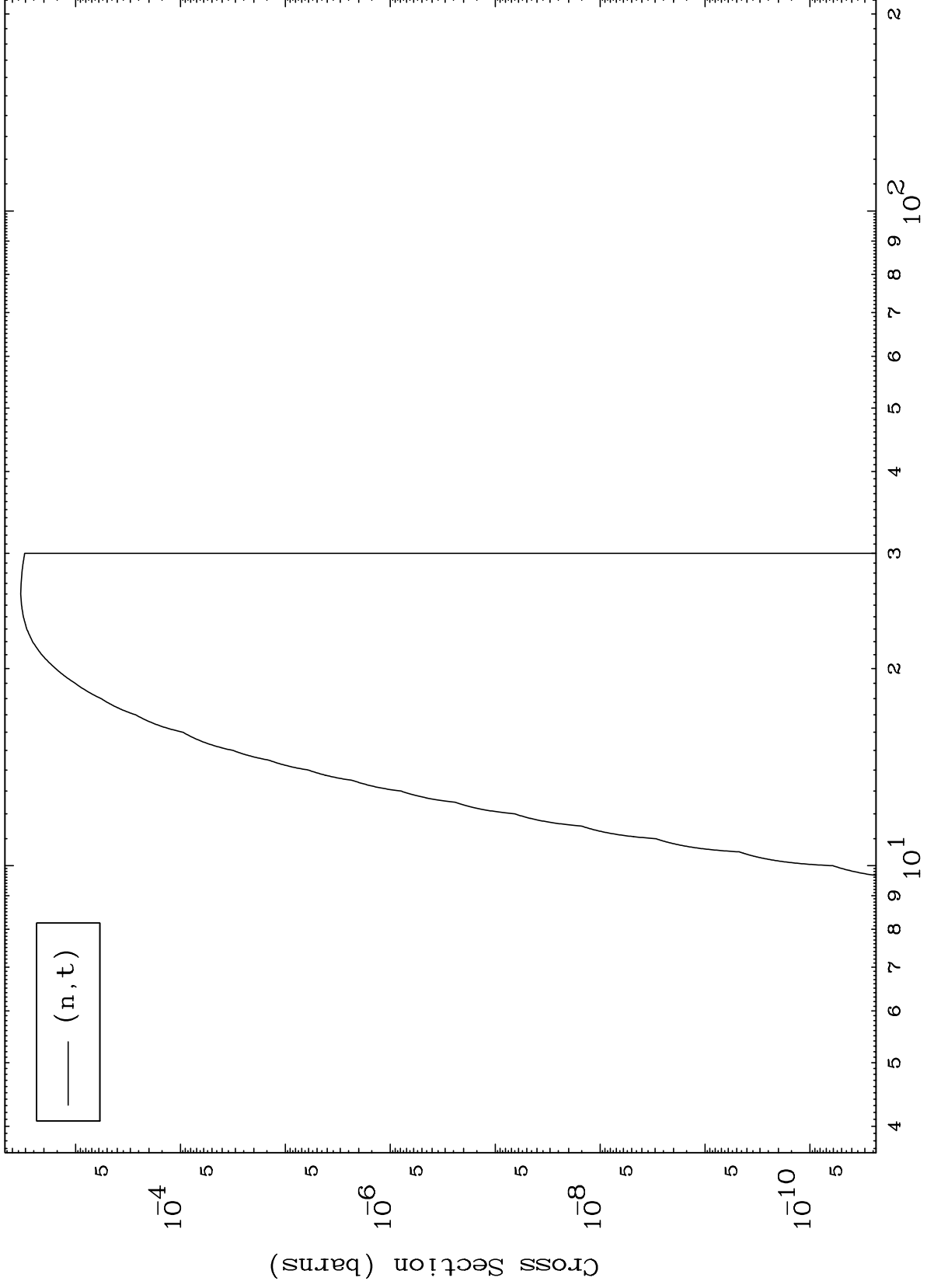
Incident Energy (MeV)

98-Cf-250

MAT 9855

(n,t) Levels
293 Kelvin Cross Sections

98-Cf-250



14

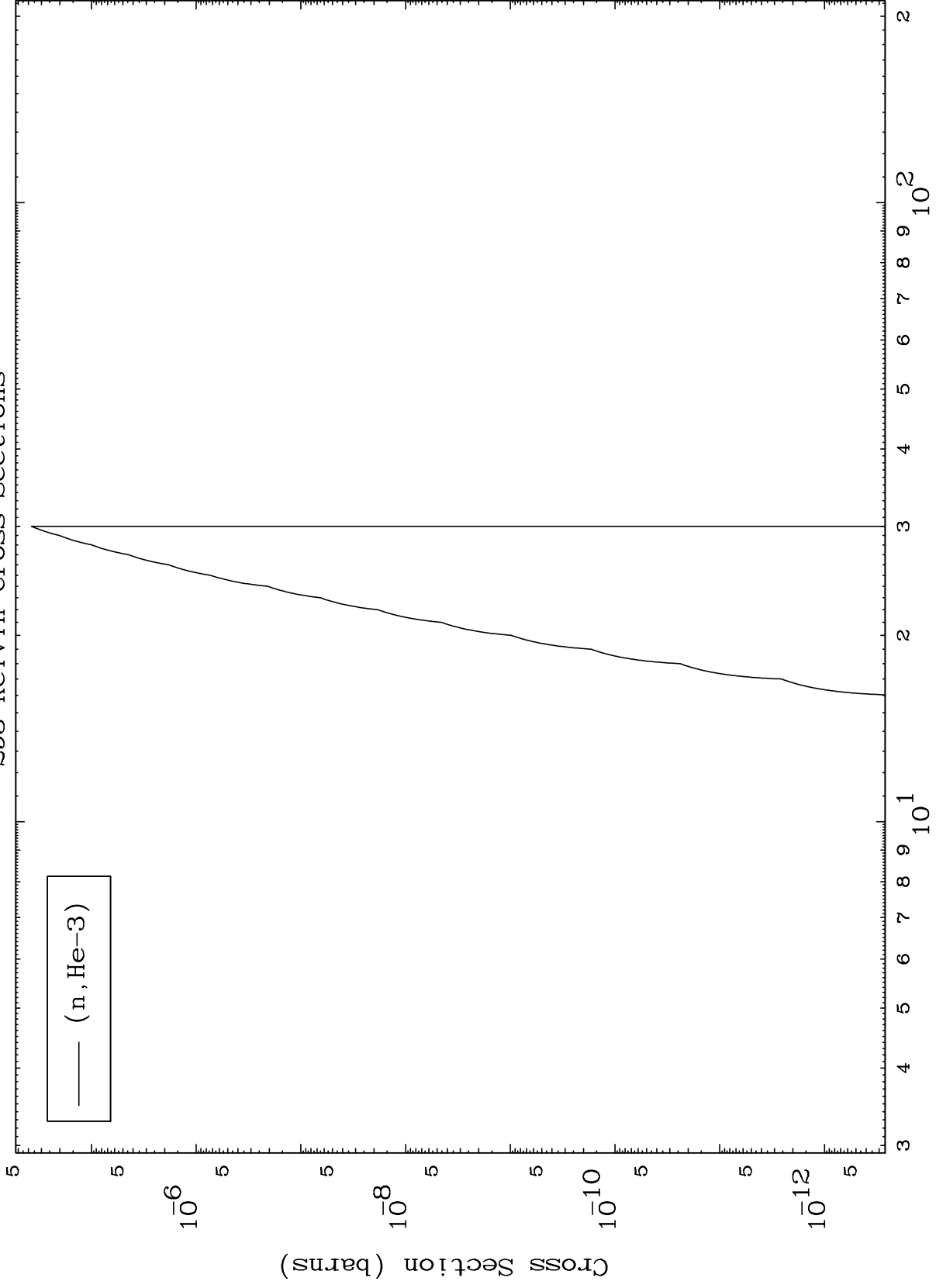
Incident Energy (MeV)

98-Cf-250

MAT 9855

(n,He3) Levels
293 Kelvin Cross Sections

98-Cf-250



15

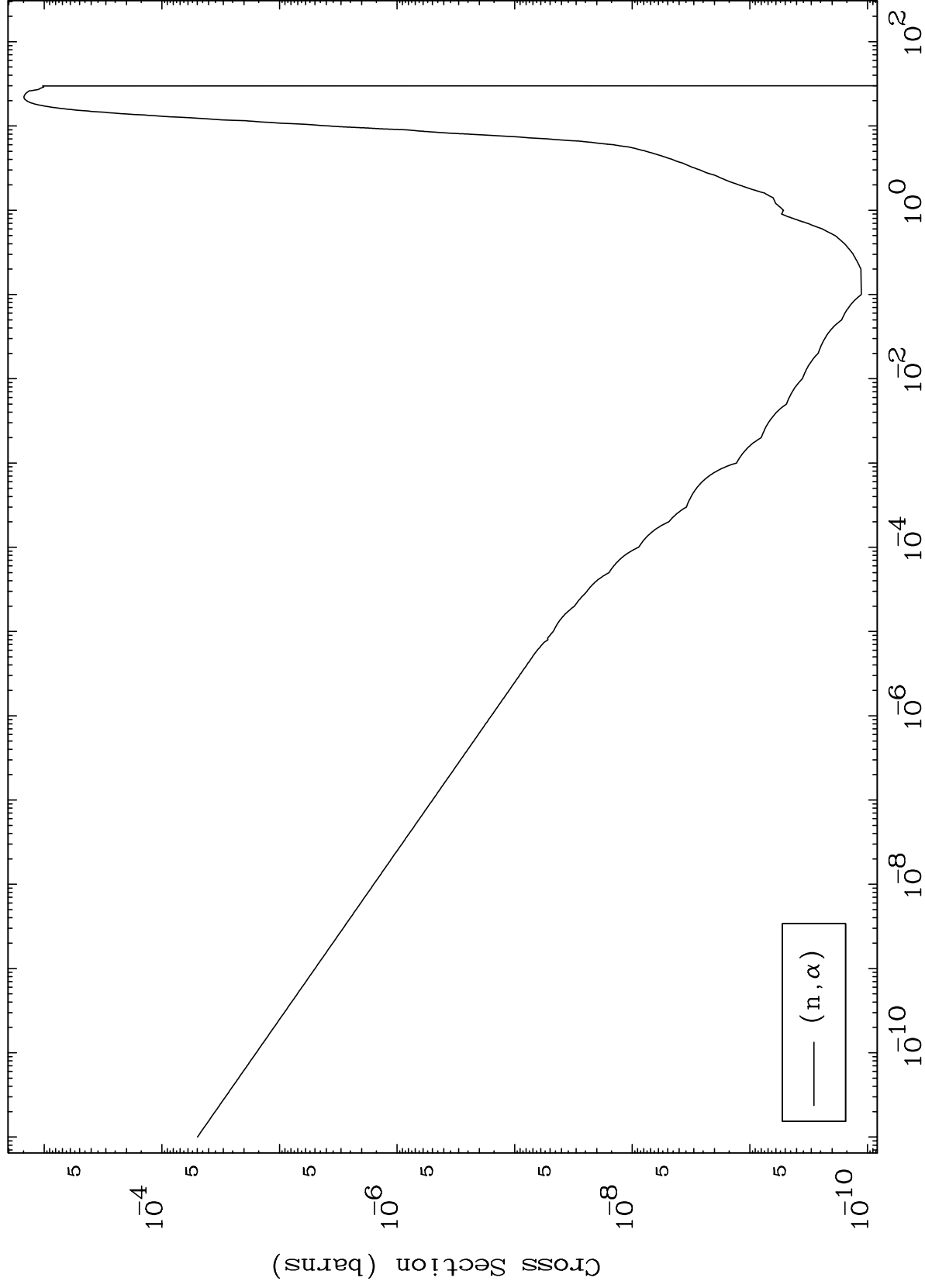
Incident Energy (MeV)

98-Cf-250

MAT 9855

(n,α) Levels
293 Kelvin Cross Sections

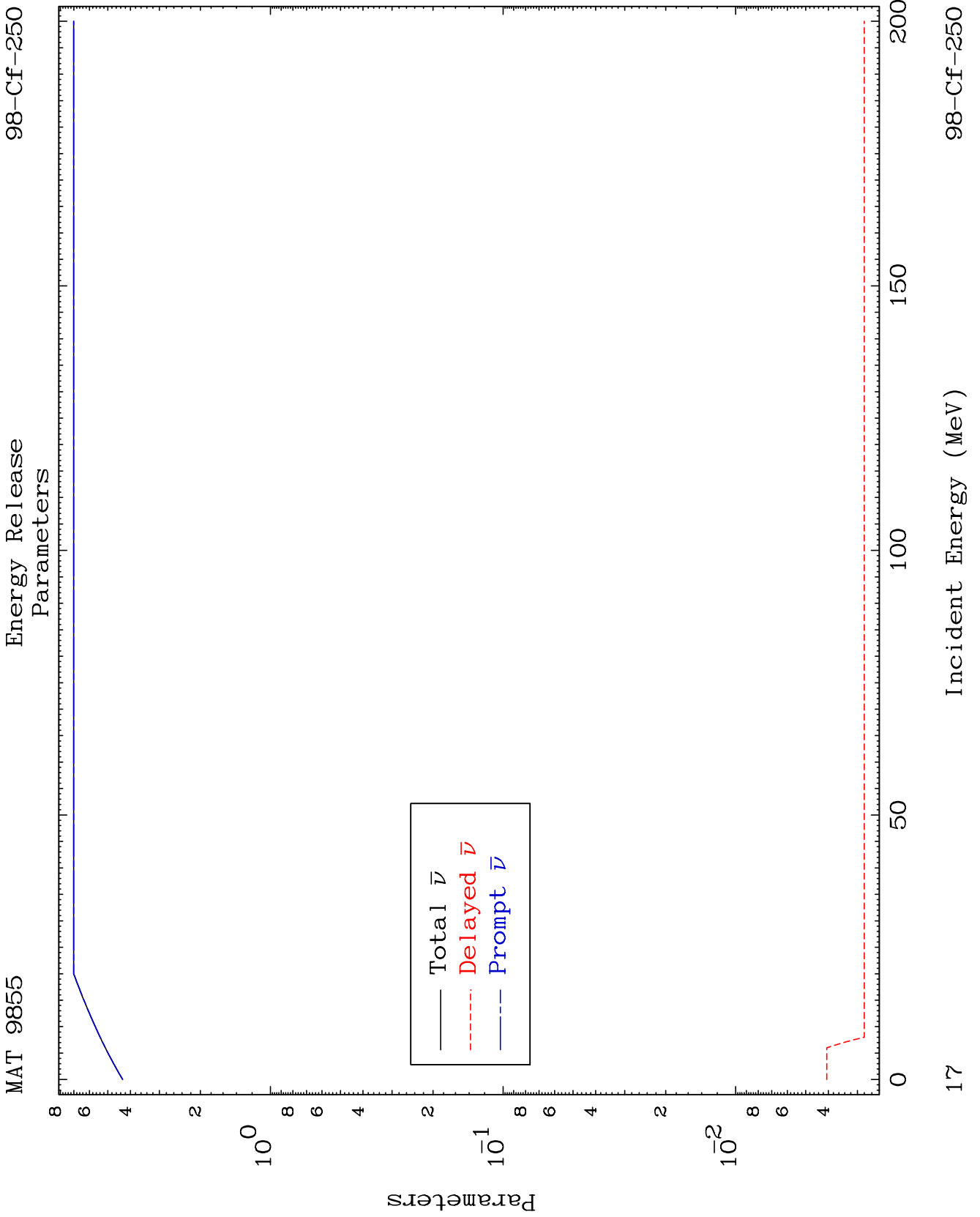
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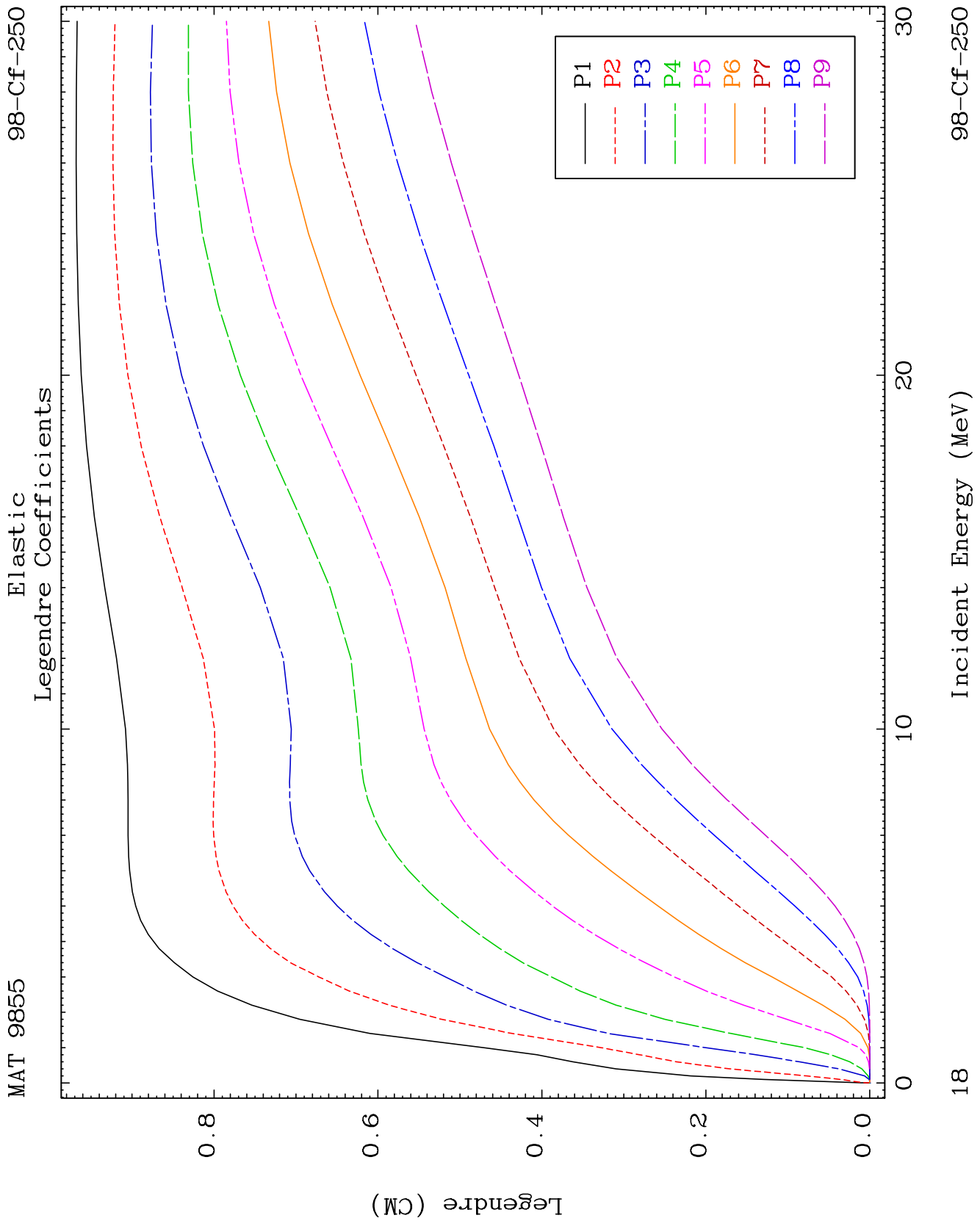


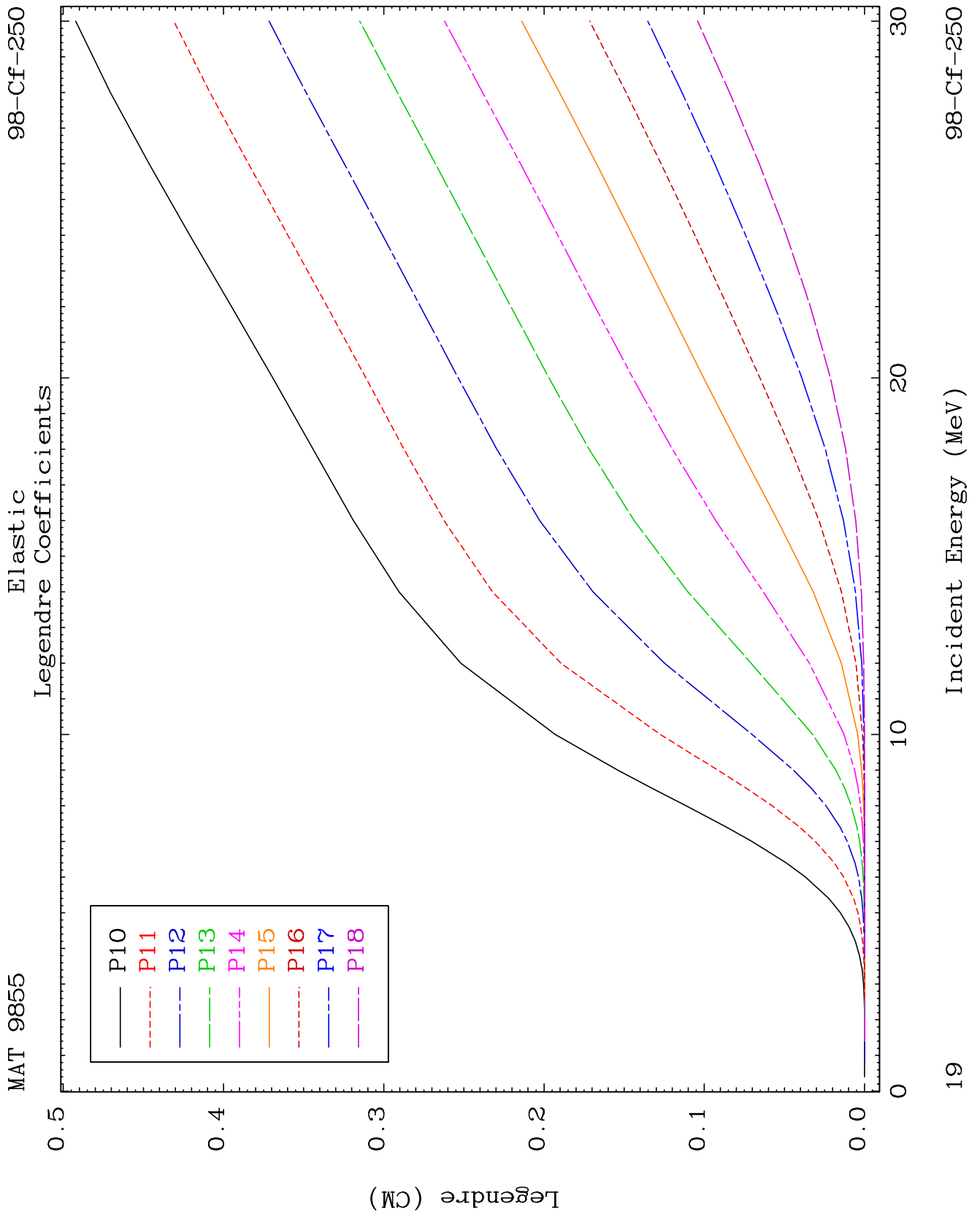
16

Incident Energy (MeV)

98-Cf-250



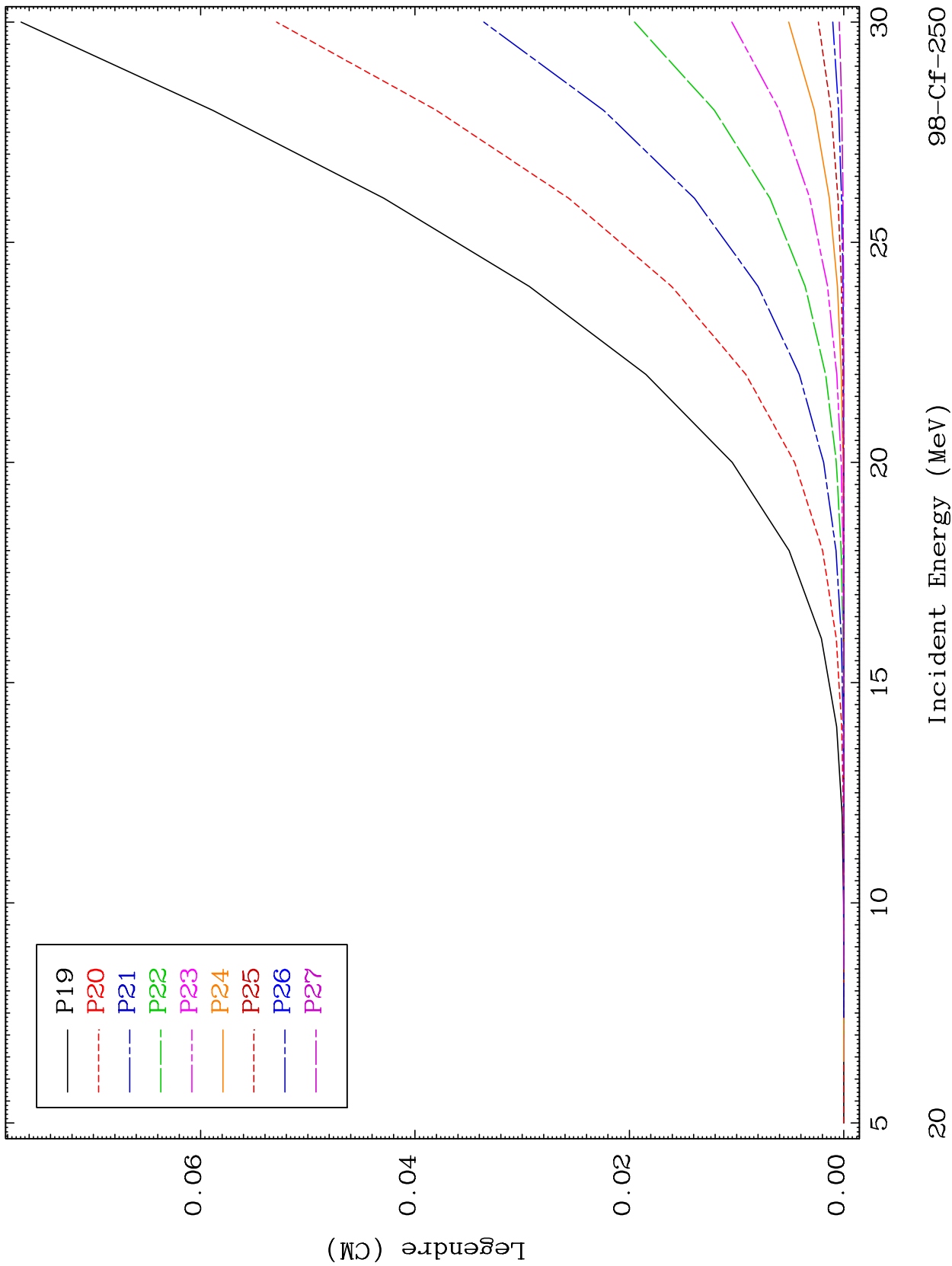


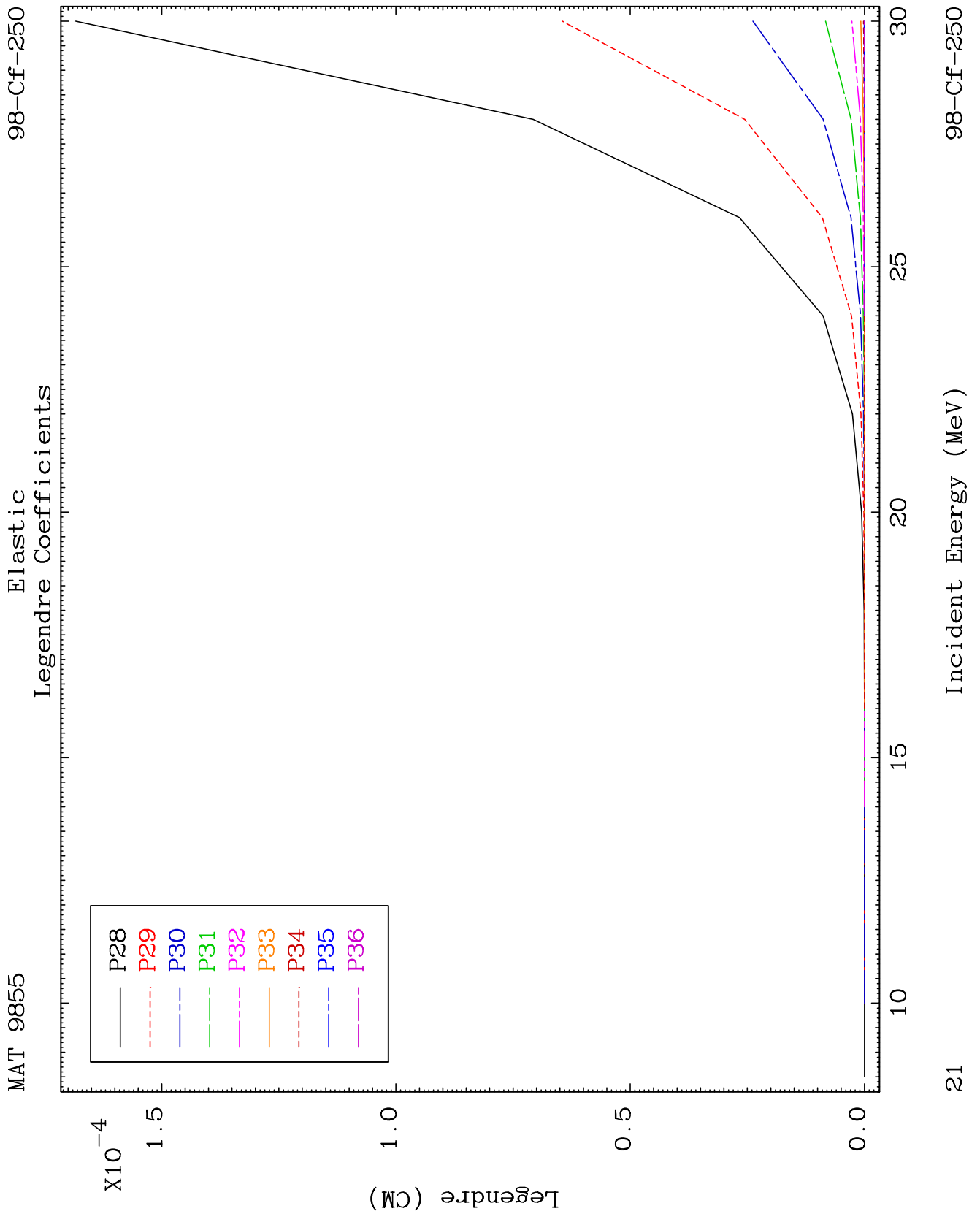


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Elastic Legendre Coefficients

98-Cf-250

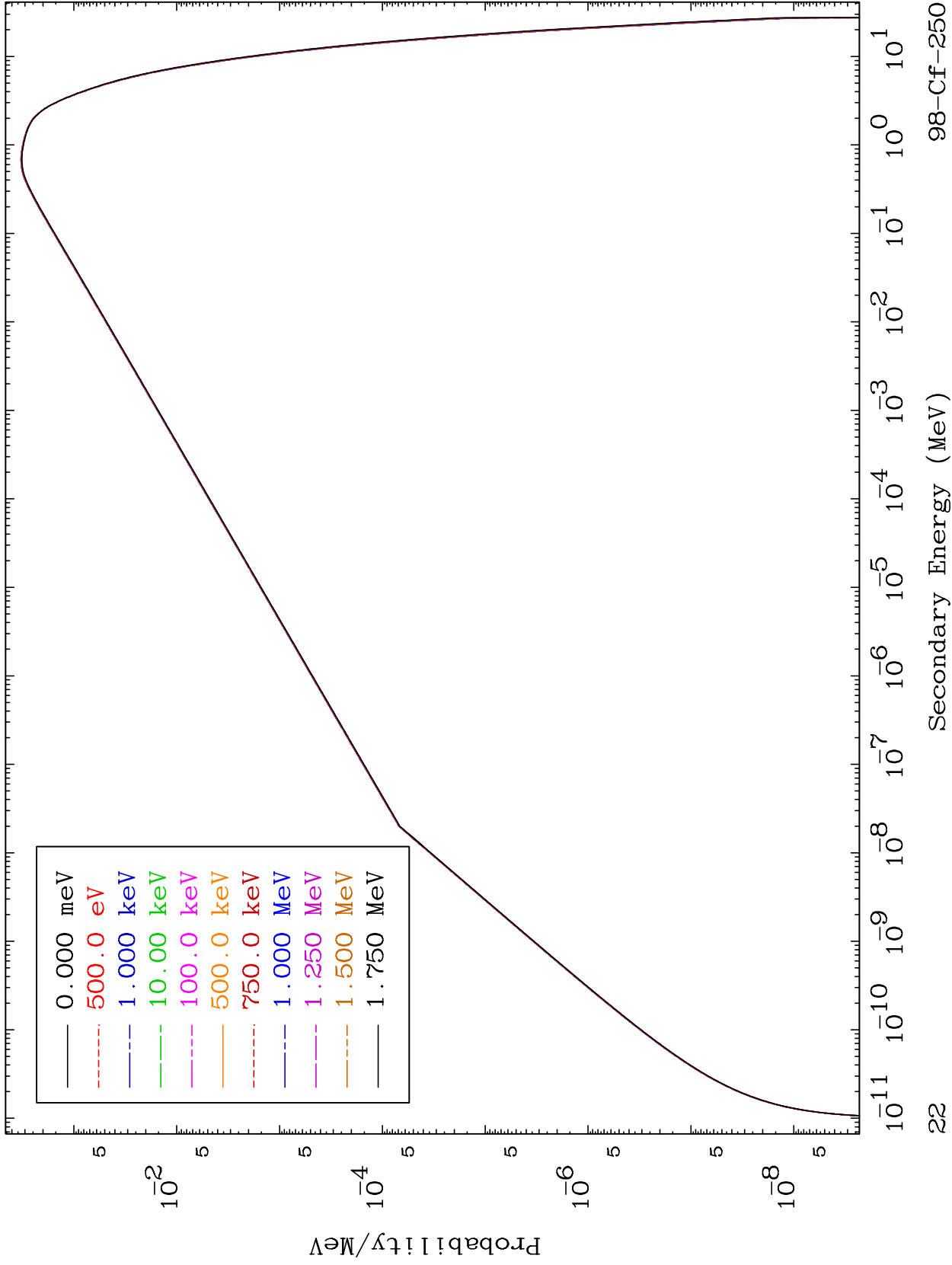




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Fission Energy Distributions

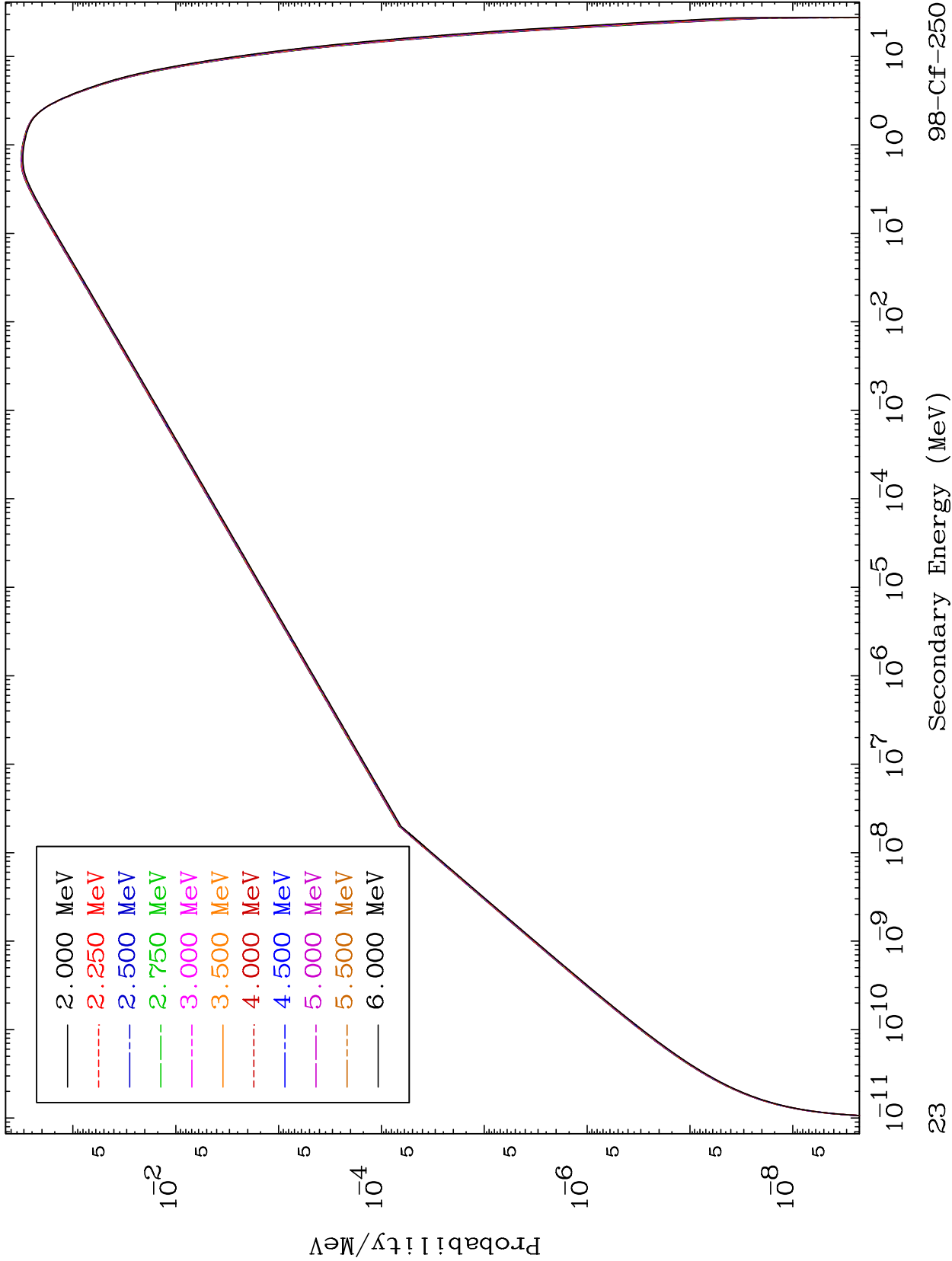
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Fission Energy Distributions

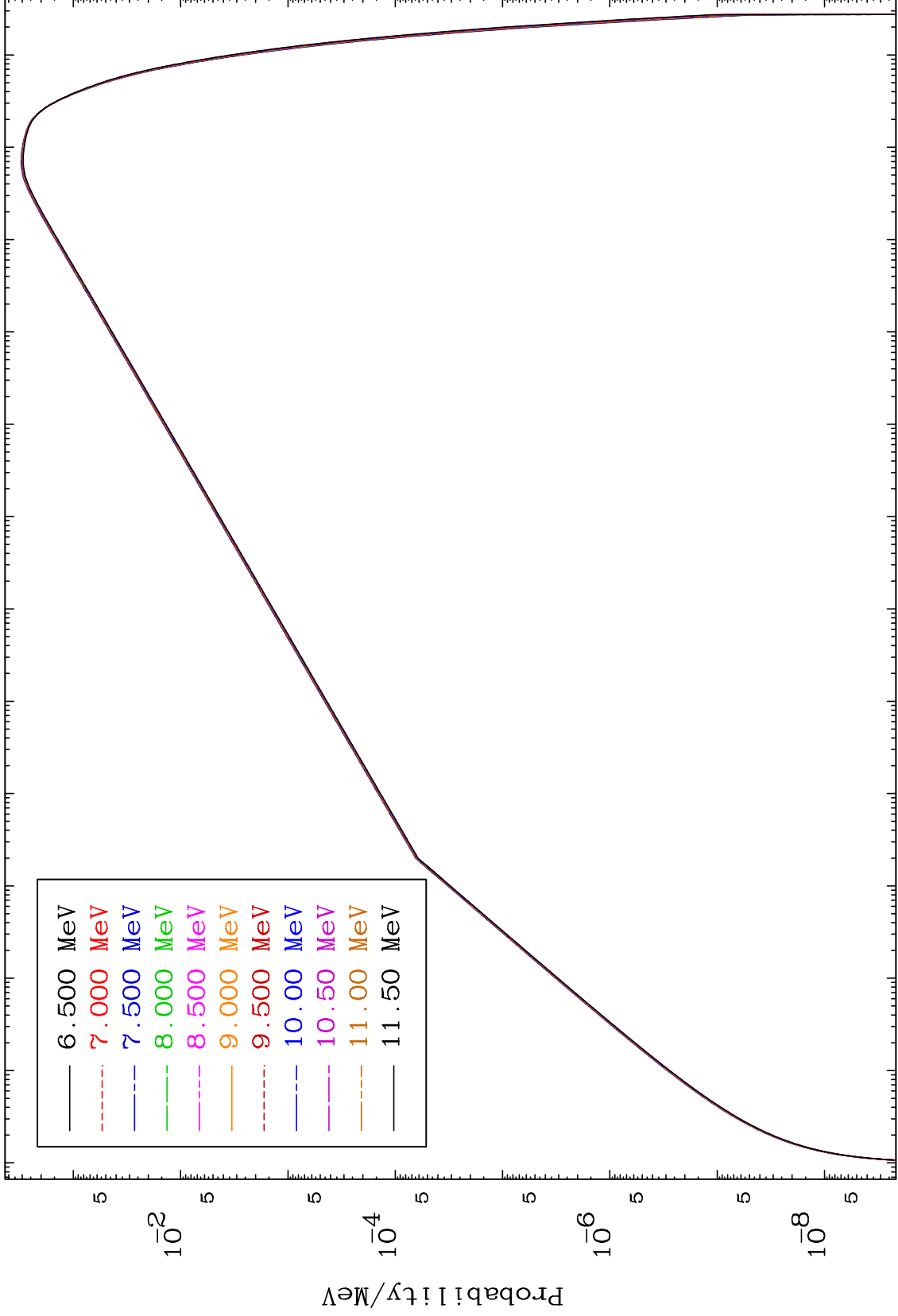
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Fission Energy Distributions

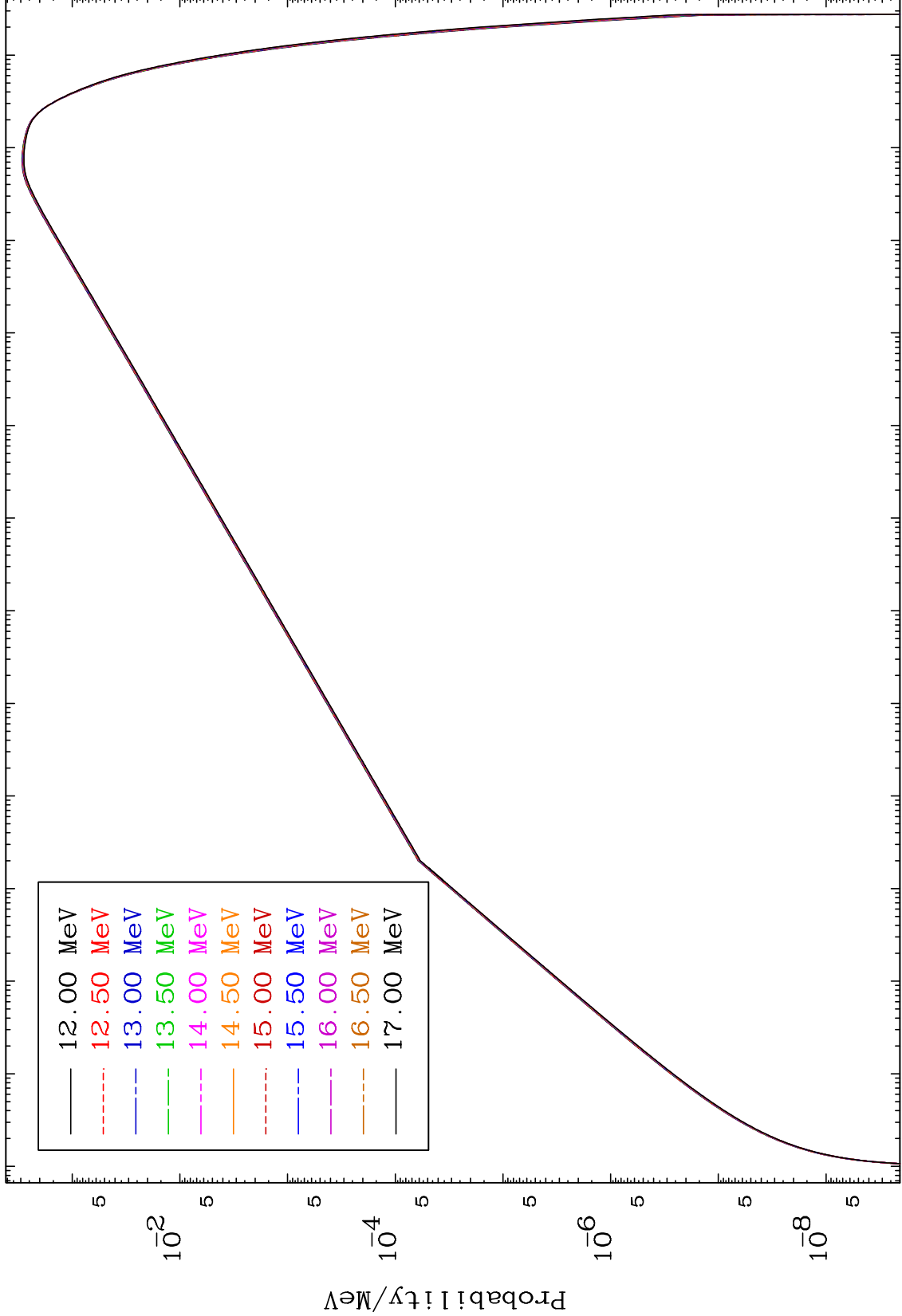
98-Cf-250



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Fission Energy Distributions

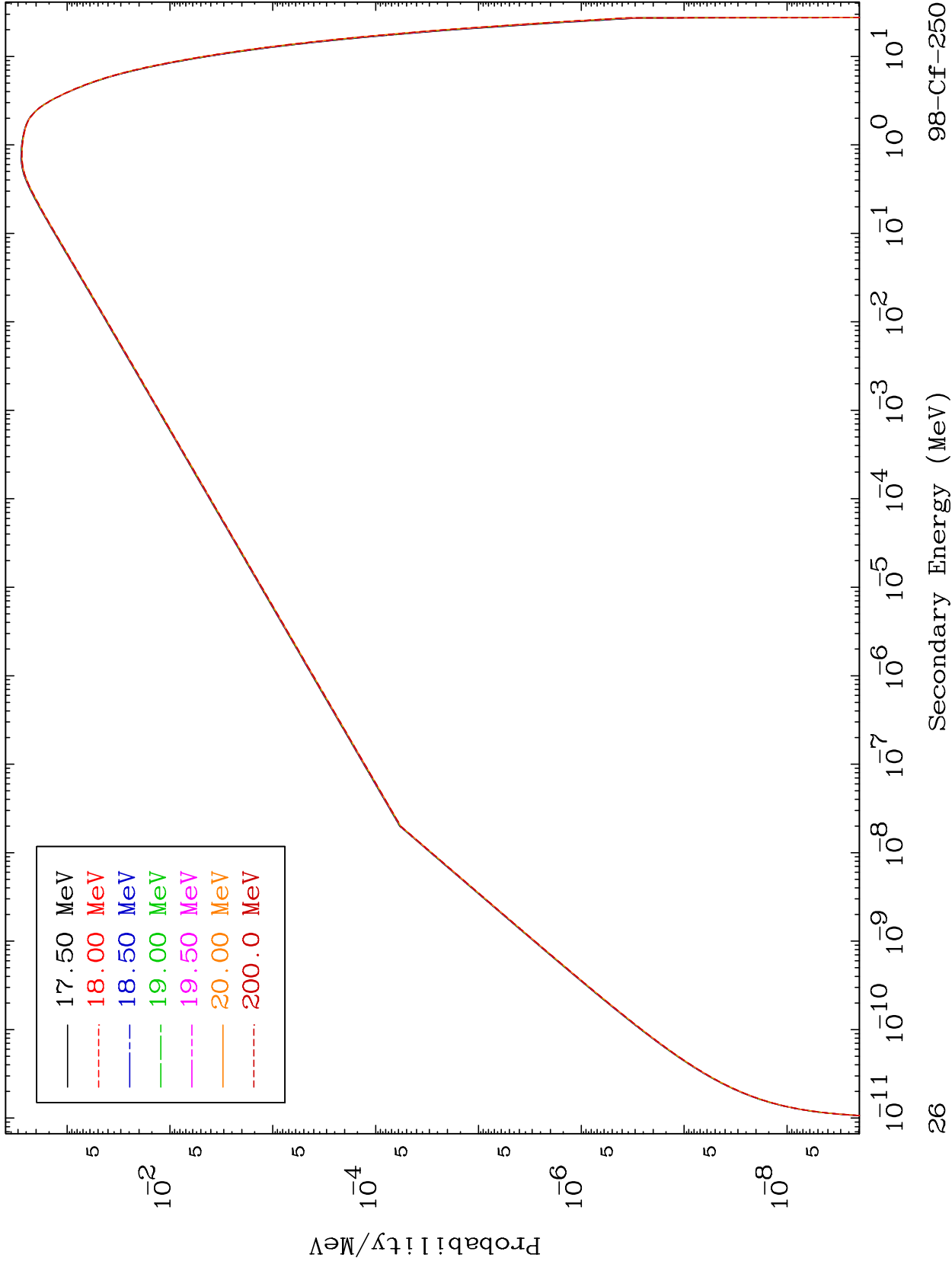
98-Cf-250



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Fission Energy Distributions

98-Cf-250

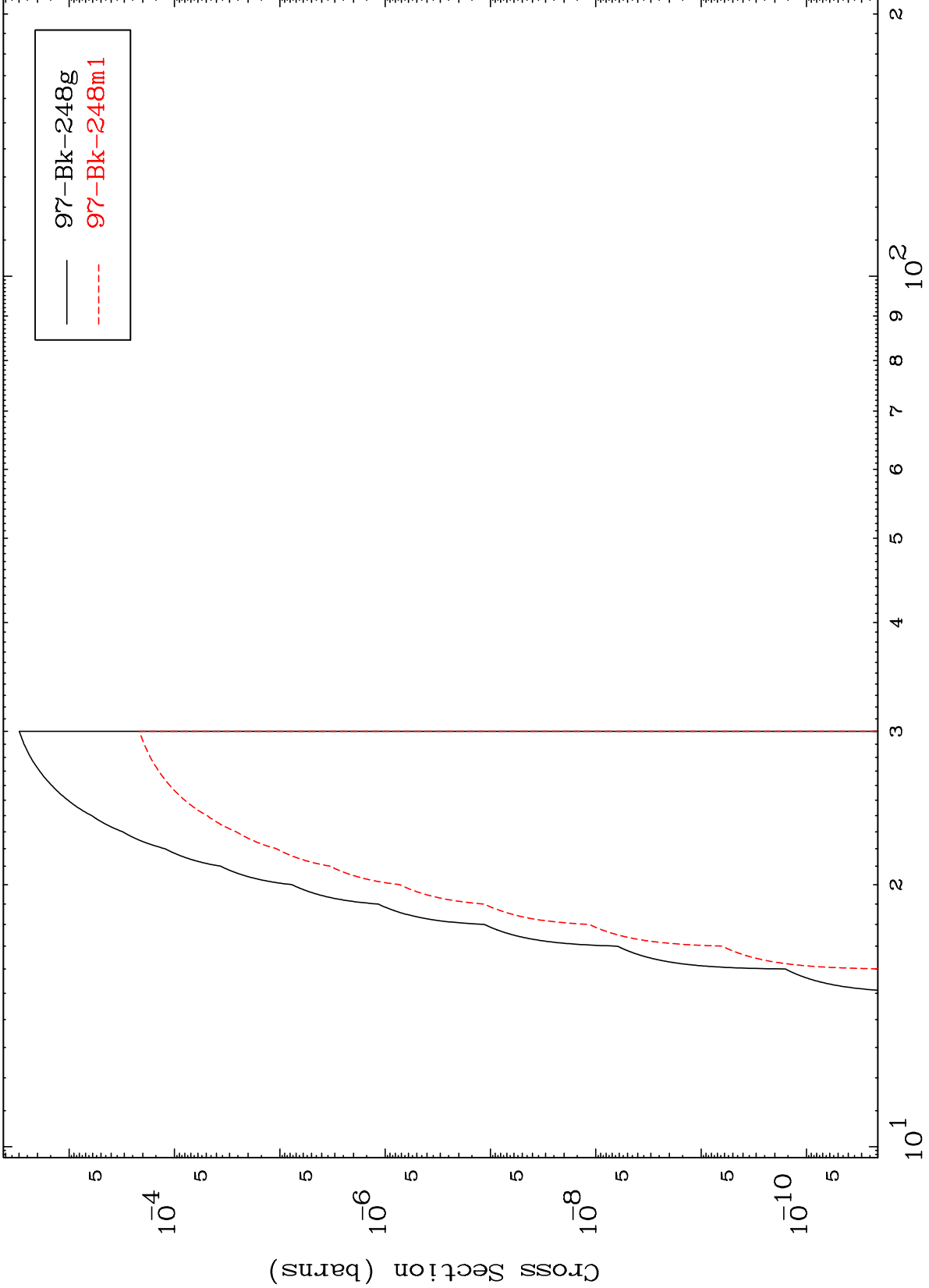


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(n,n') d

98-Cf-250

Radionuclide Production Cross Section



97-Bk-248g
97-Bk-248m1

27

Incident Energy (MeV)

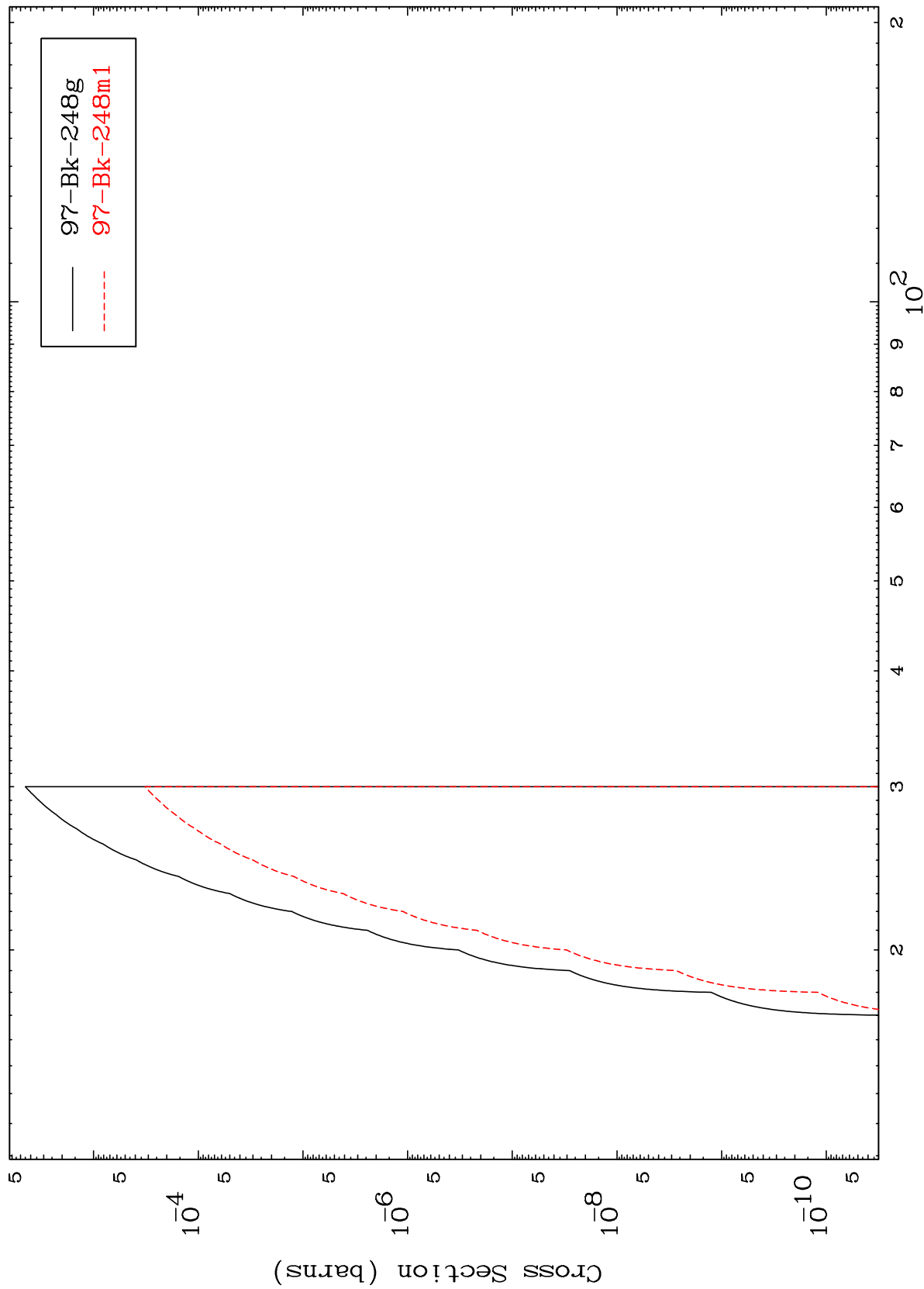
98-Cf-250

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(n,2n) p

98-Cf-250

Radionuclide Production Cross Section



28

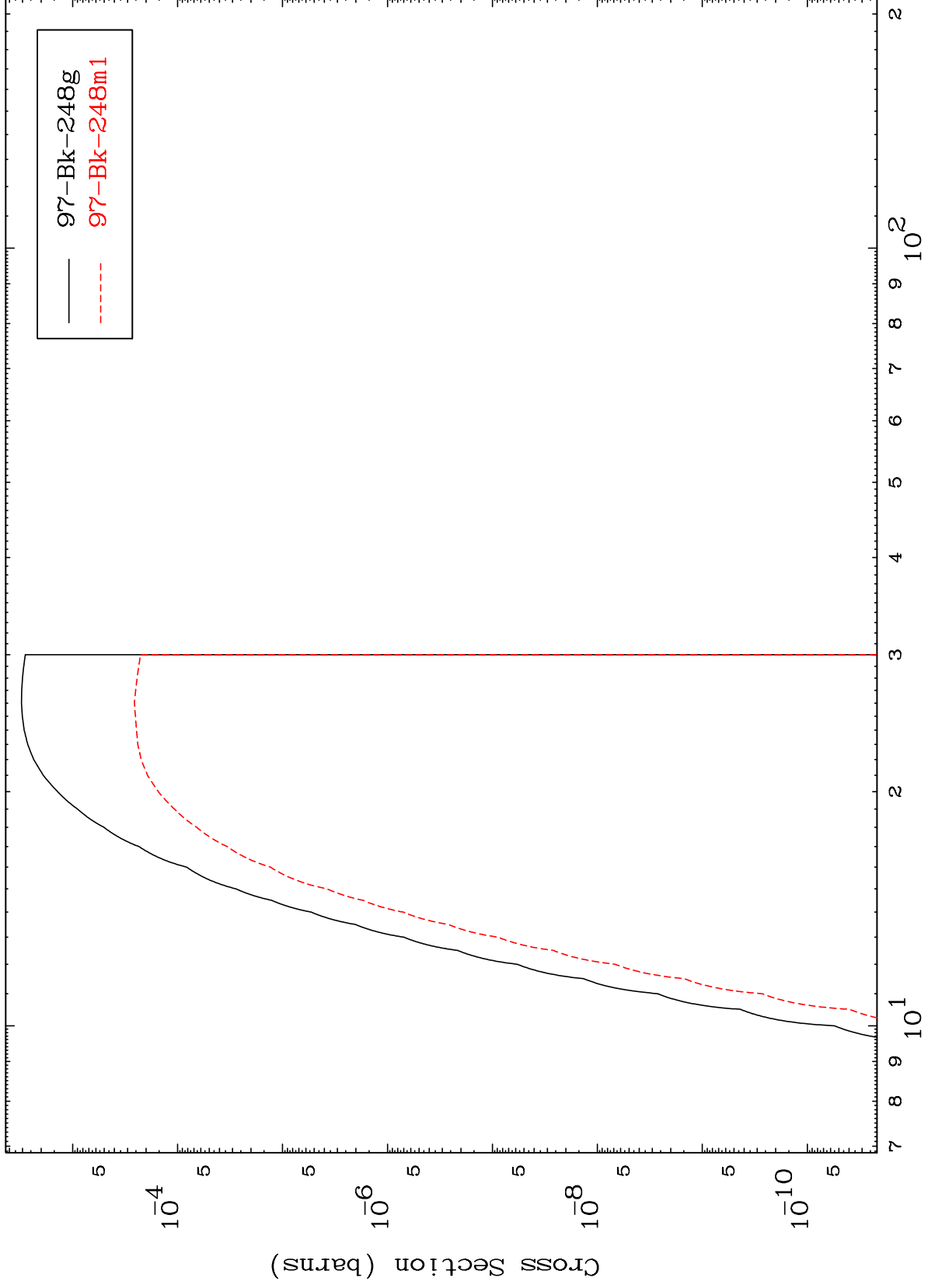
Incident Energy (MeV)

98-Cf-250

MAT 9855

98-Cf-250

(n,t)
Radionuclide Production Cross Section



29

Incident Energy (MeV)

98-Cf-250