

Program EVALPLOT
(Version 2021-1)

by

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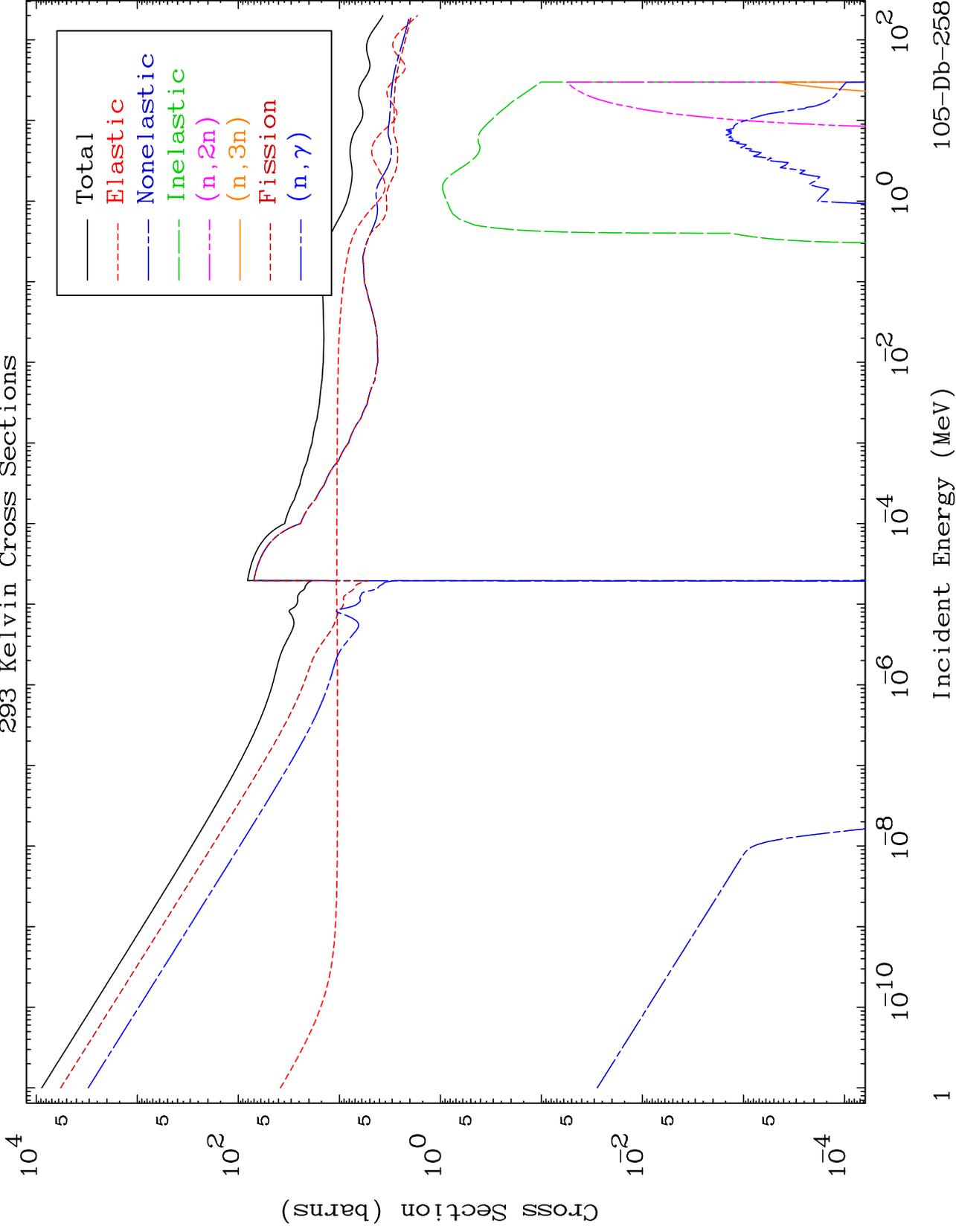
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Press Mouse Button to Start

MAT 558

Neutron Major
293 Kelvin Cross Sections

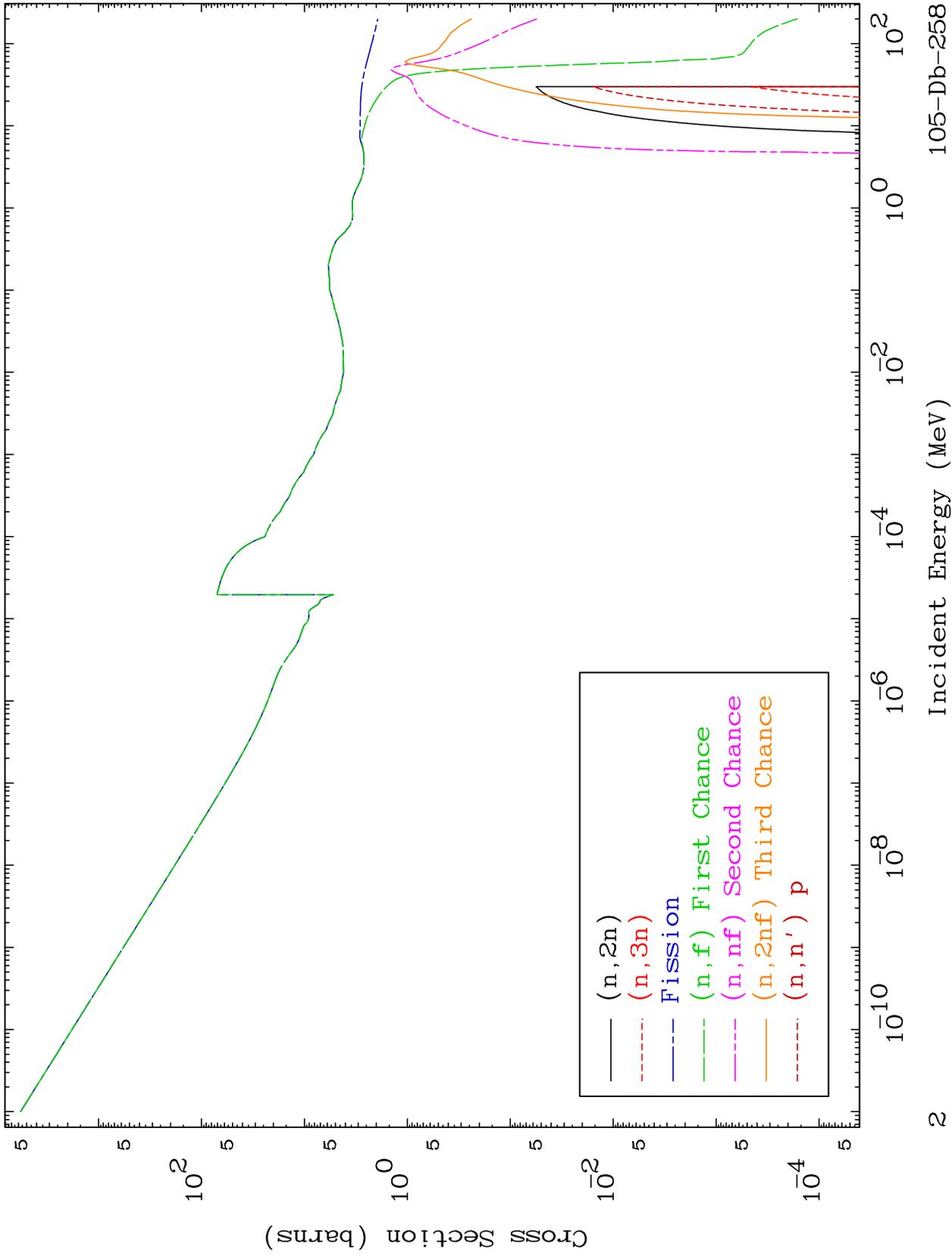
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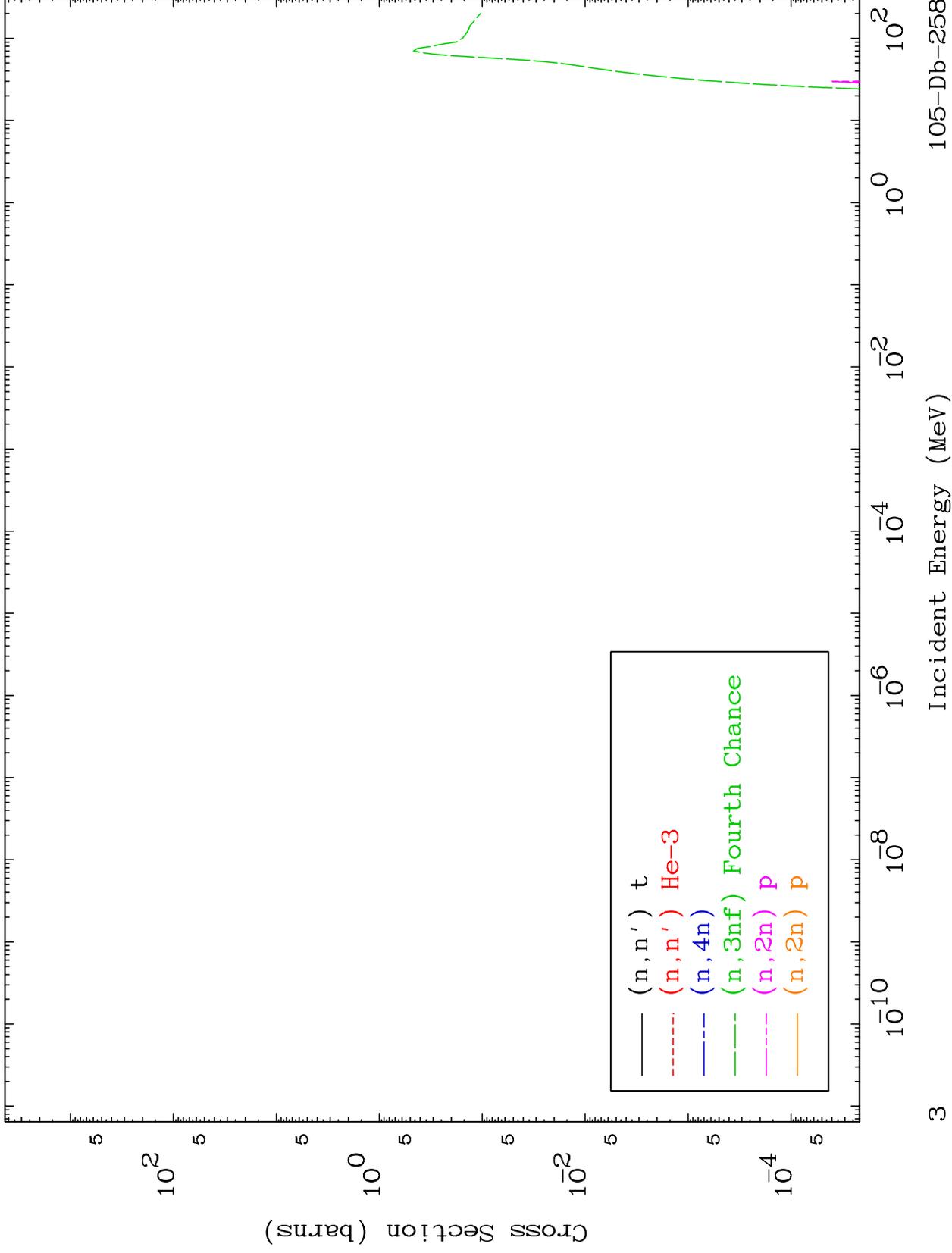


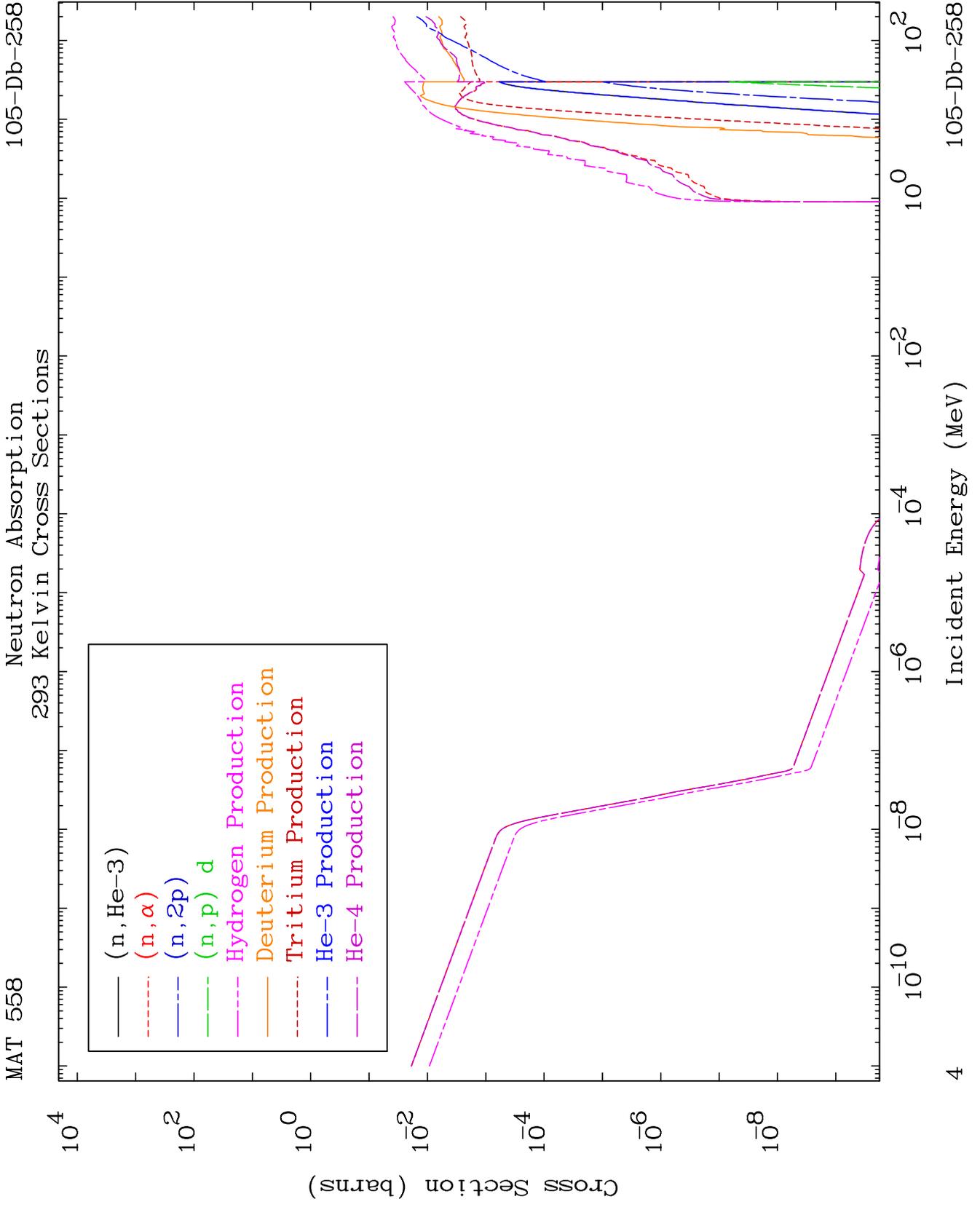
MAT 558

Neutron Absorption
293 Kelvin Cross Sections

105-Db-258



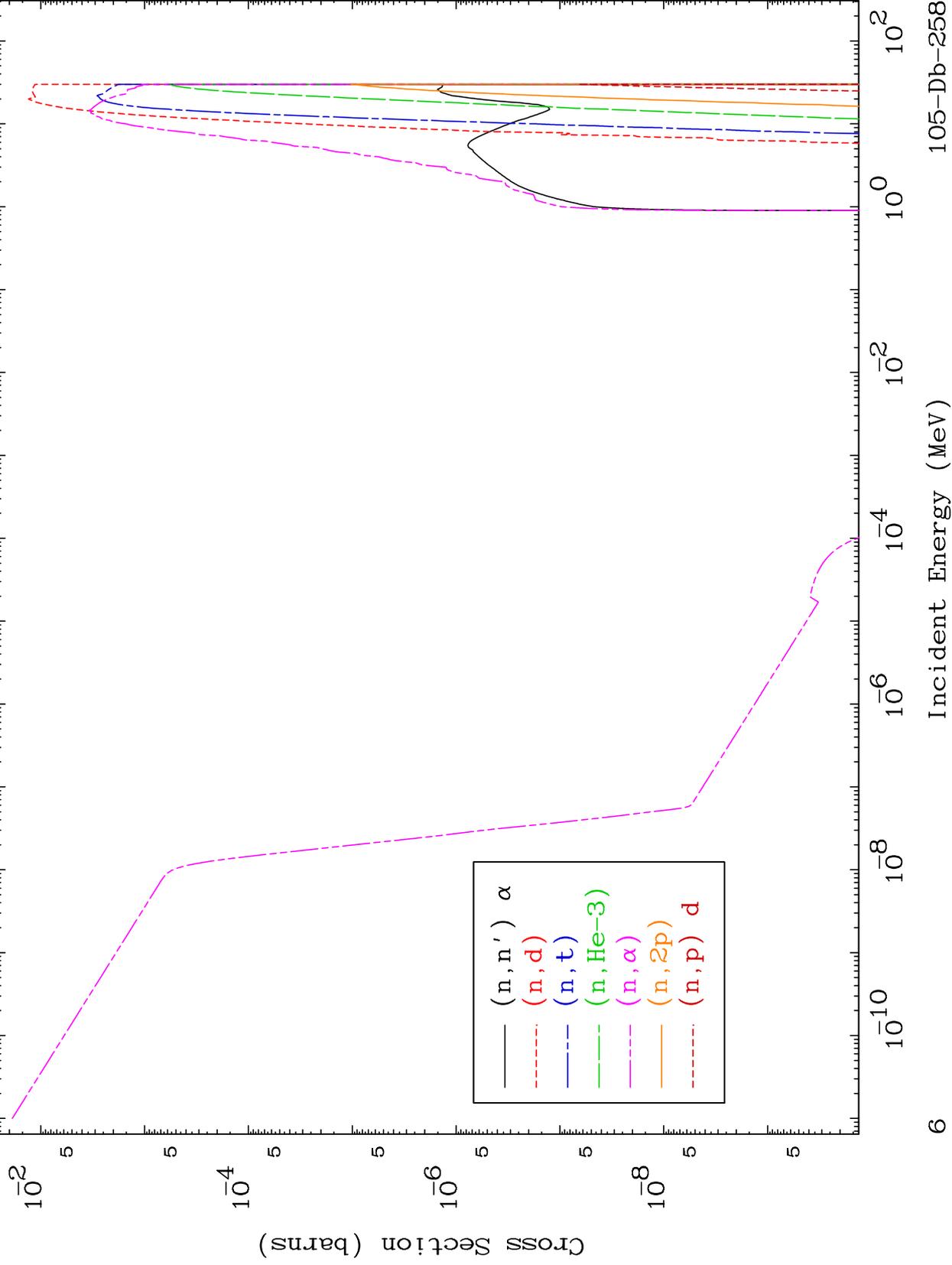




MAT 558

Charged Particle
293 Kelvin Cross Sections

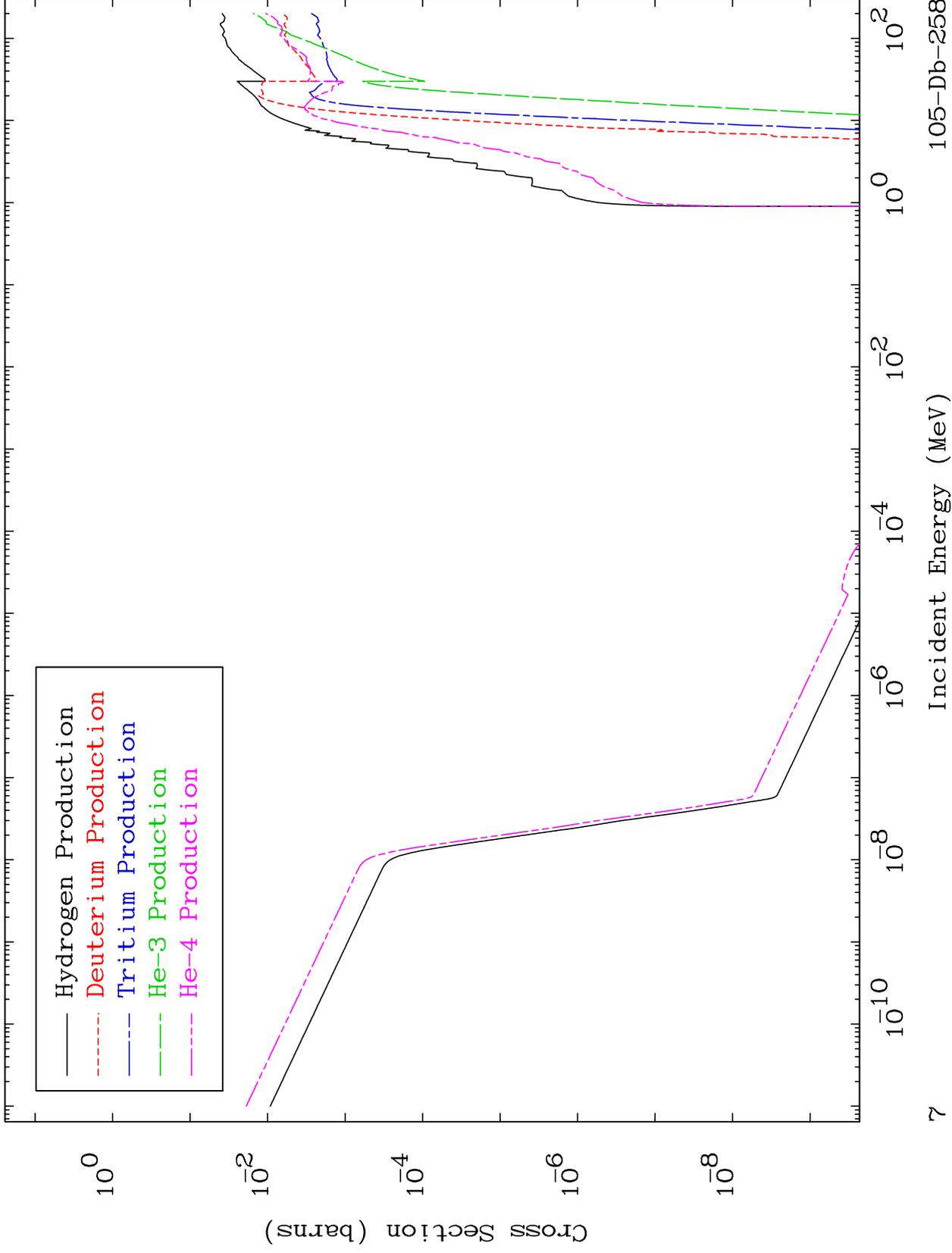
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MAT 558

Particle Production
293 Kelvin Cross Sections

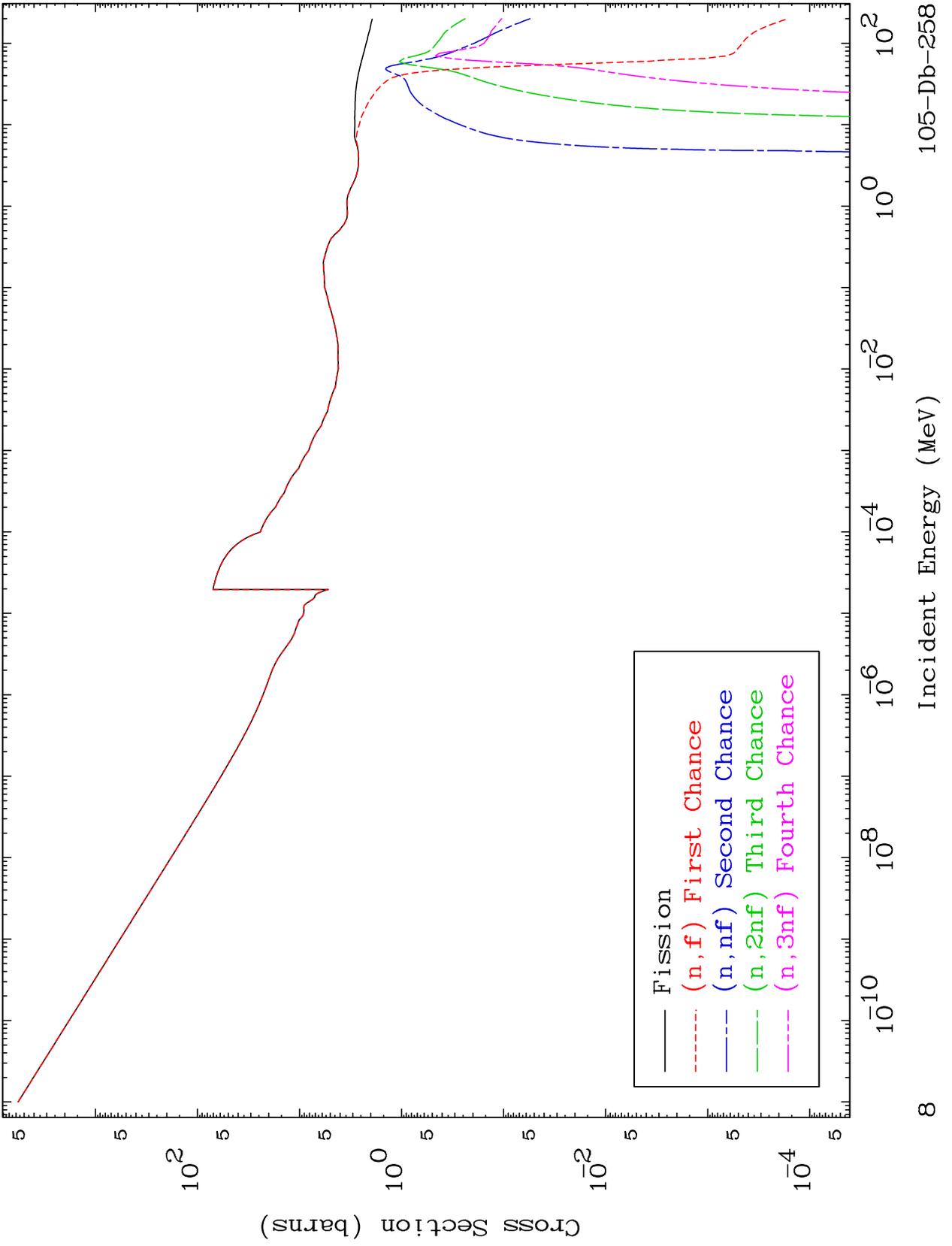
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MAT 558

Fission
293 Kelvin Cross Sections

105-Db-258

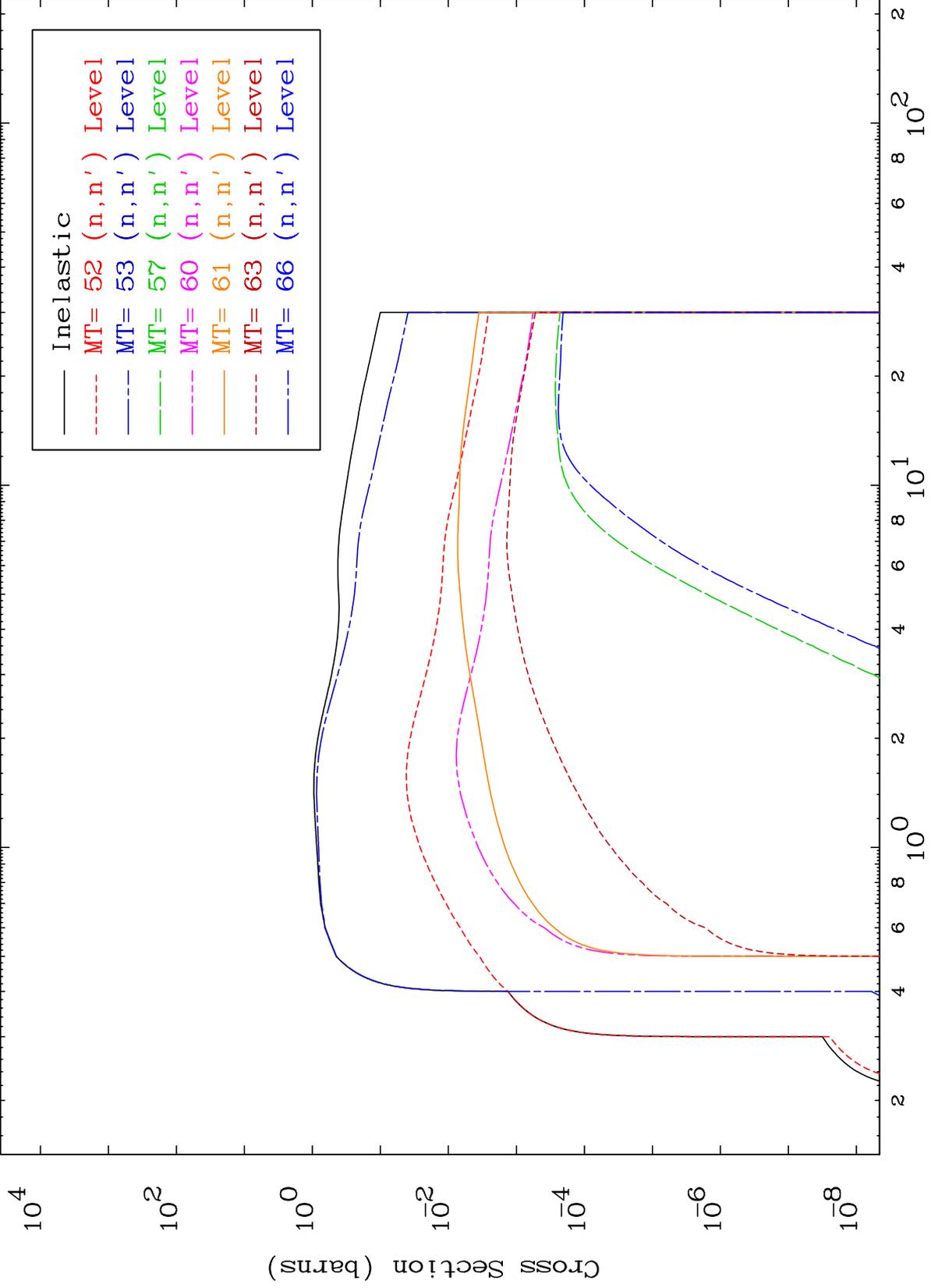


MAT 558

(n,n') Levels

105-Db-258

293 Kelvin Cross Sections

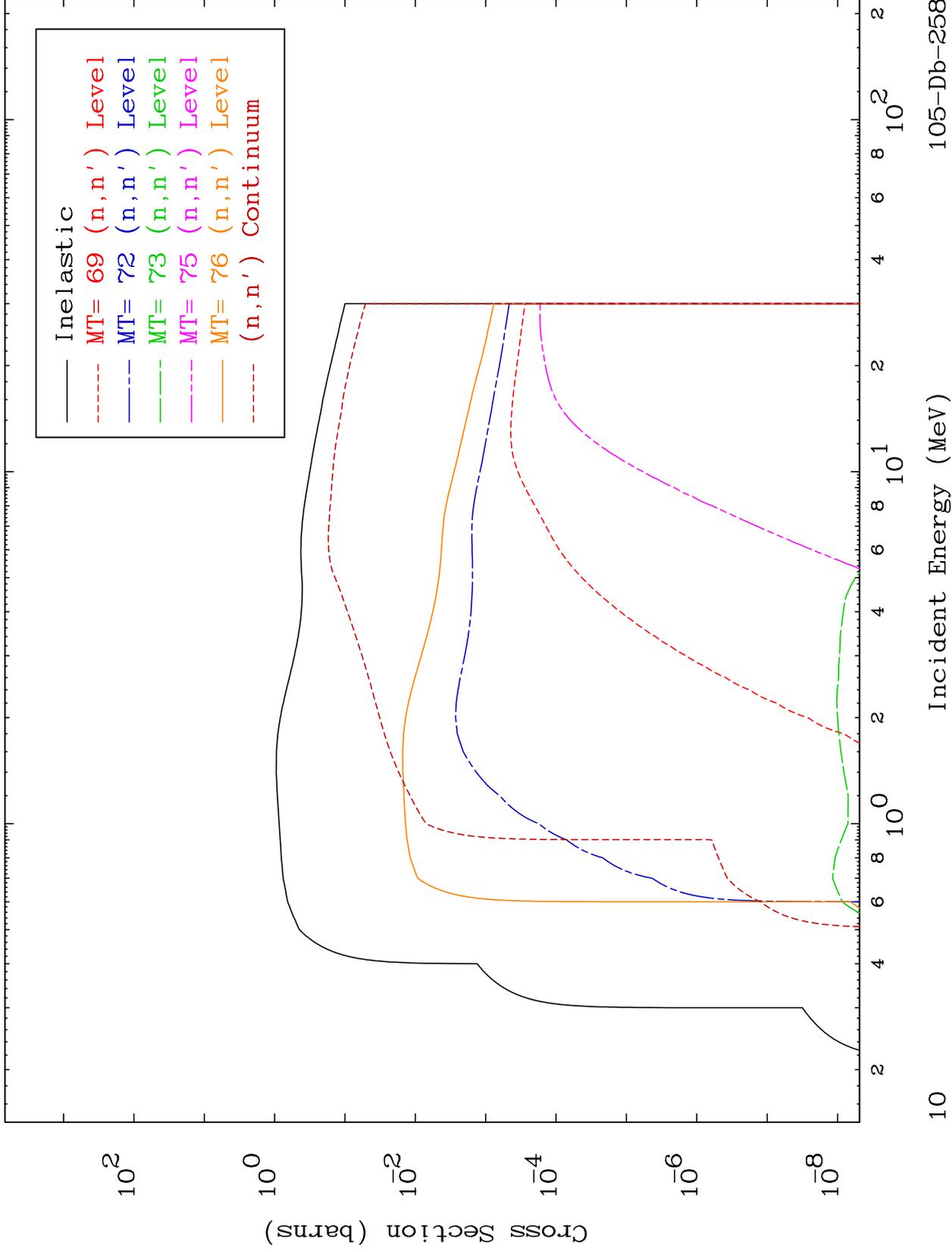


MAT 558

(n,n') Levels

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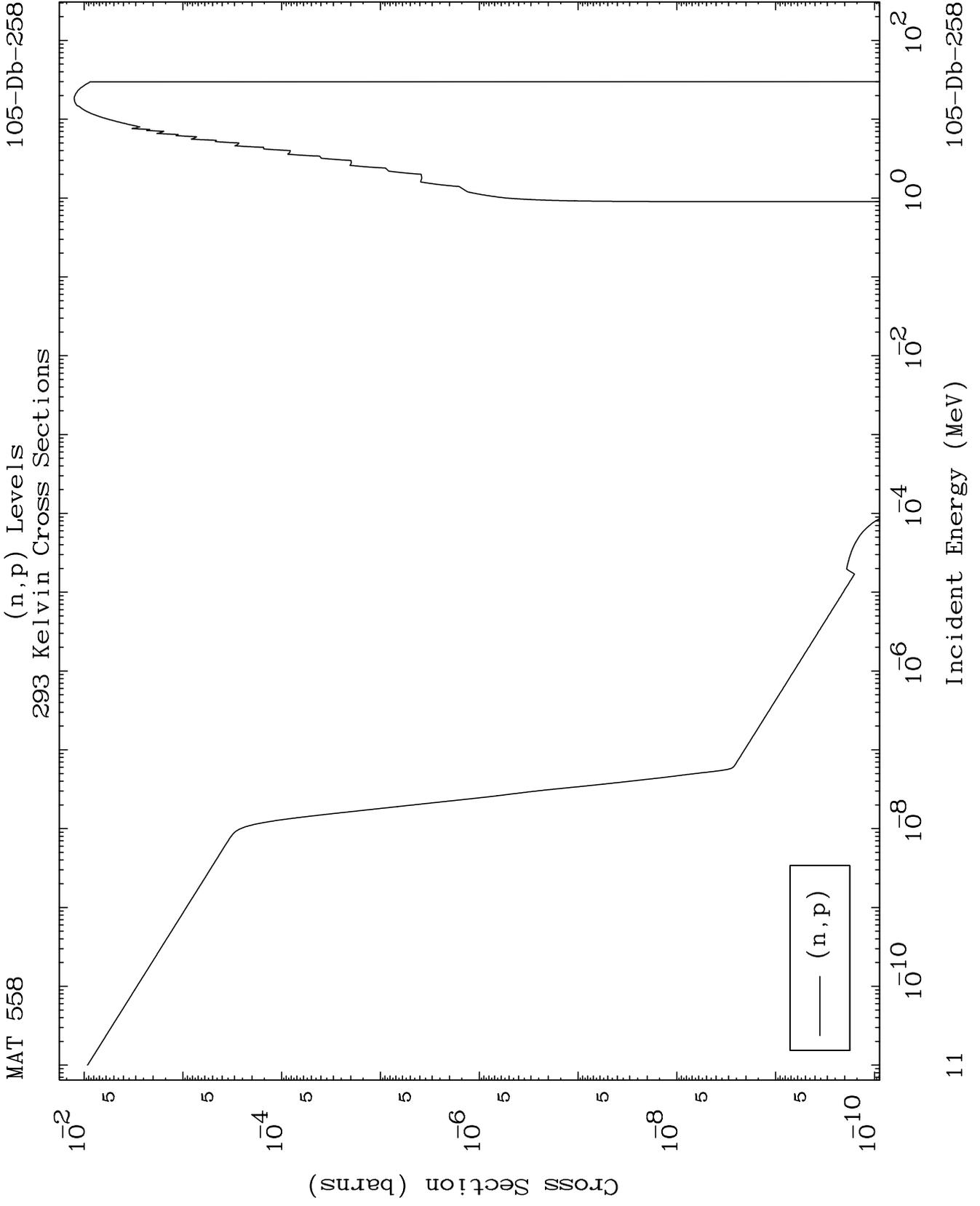
293 Kelvin Cross Sections



10

Incident Energy (MeV)

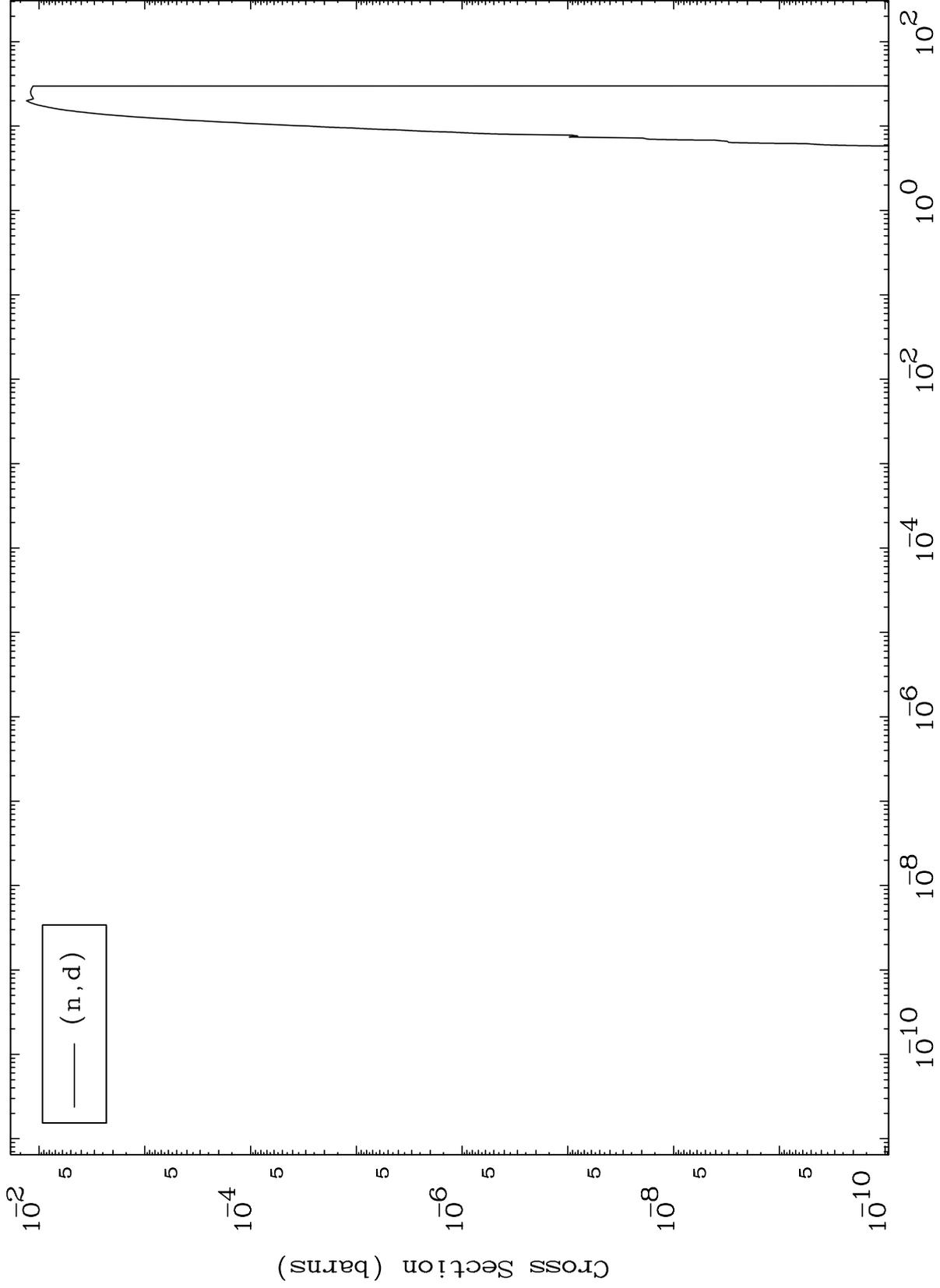
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(n,d) Levels
293 Kelvin Cross Sections

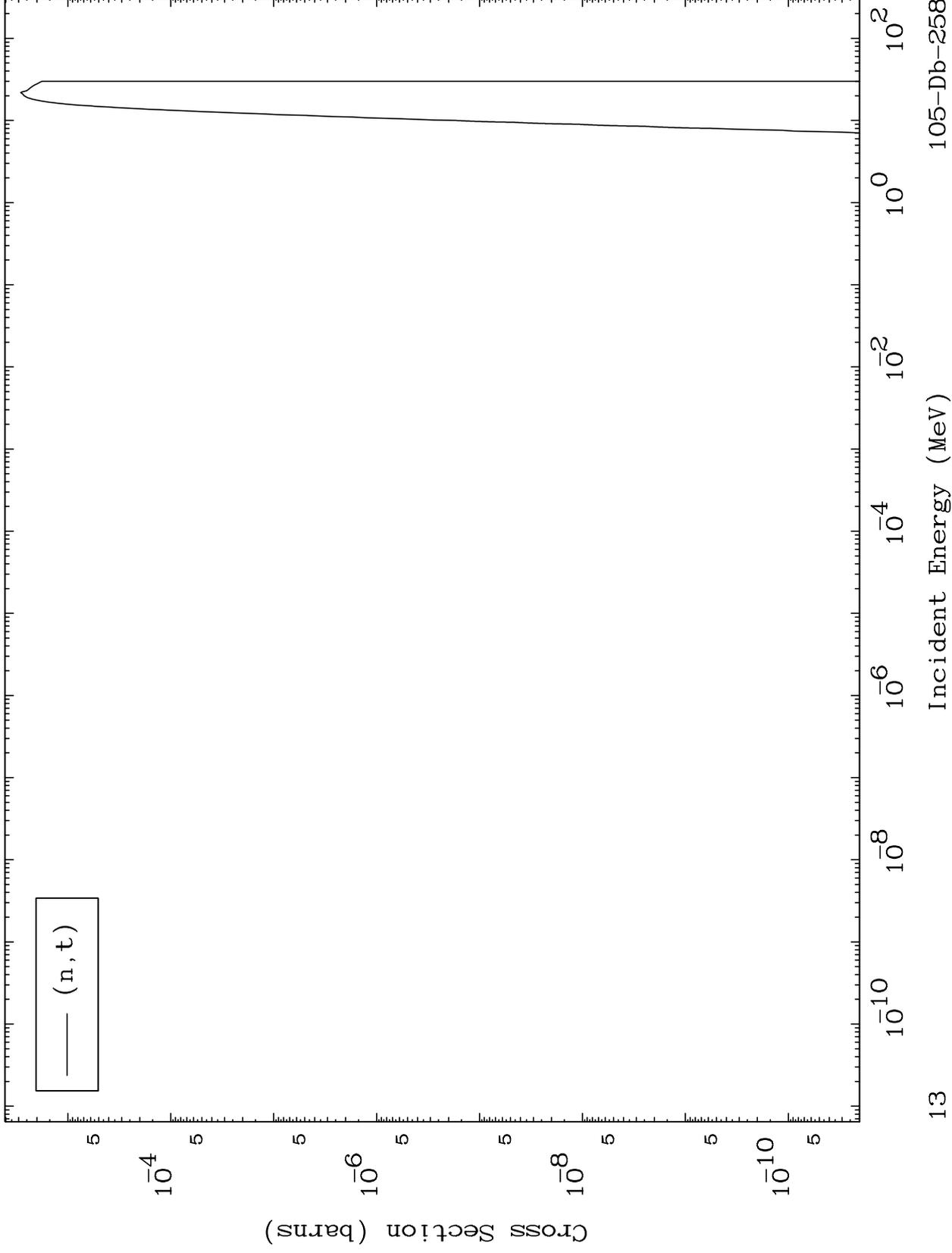
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(n,t) Levels
293 Kelvin Cross Sections

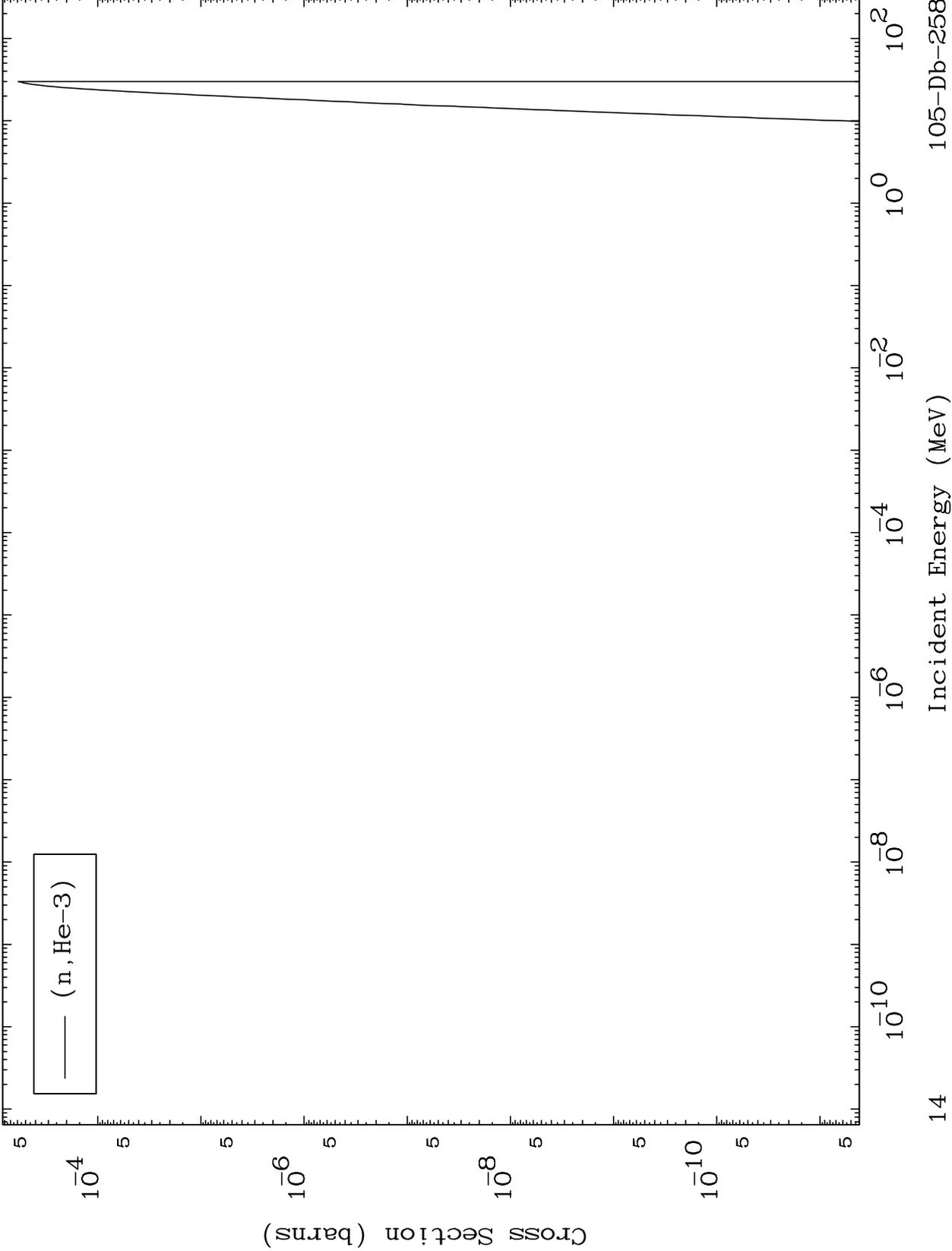
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(n,He3) Levels
293 Kelvin Cross Sections

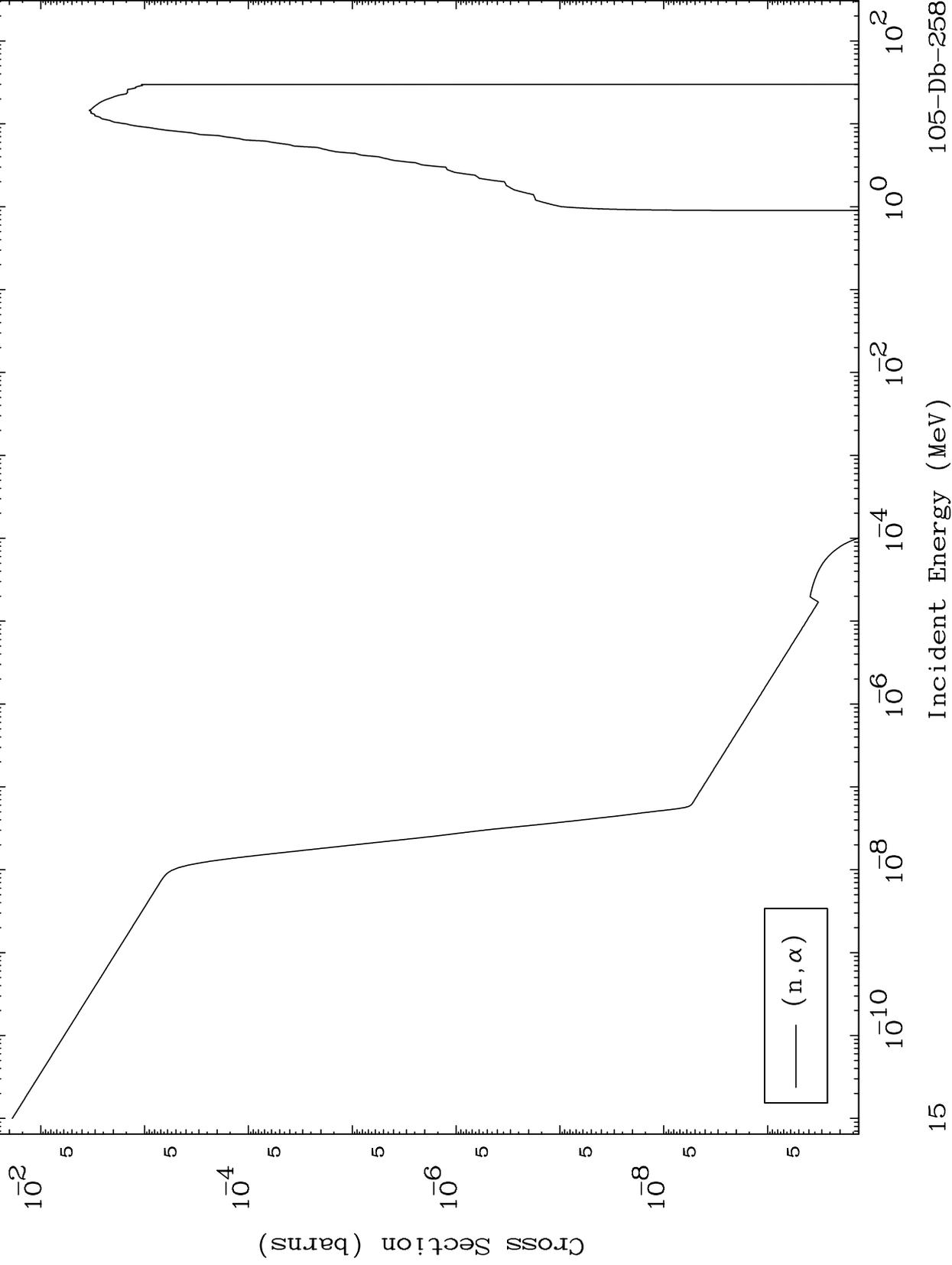
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(n,α) Levels
293 Kelvin Cross Sections

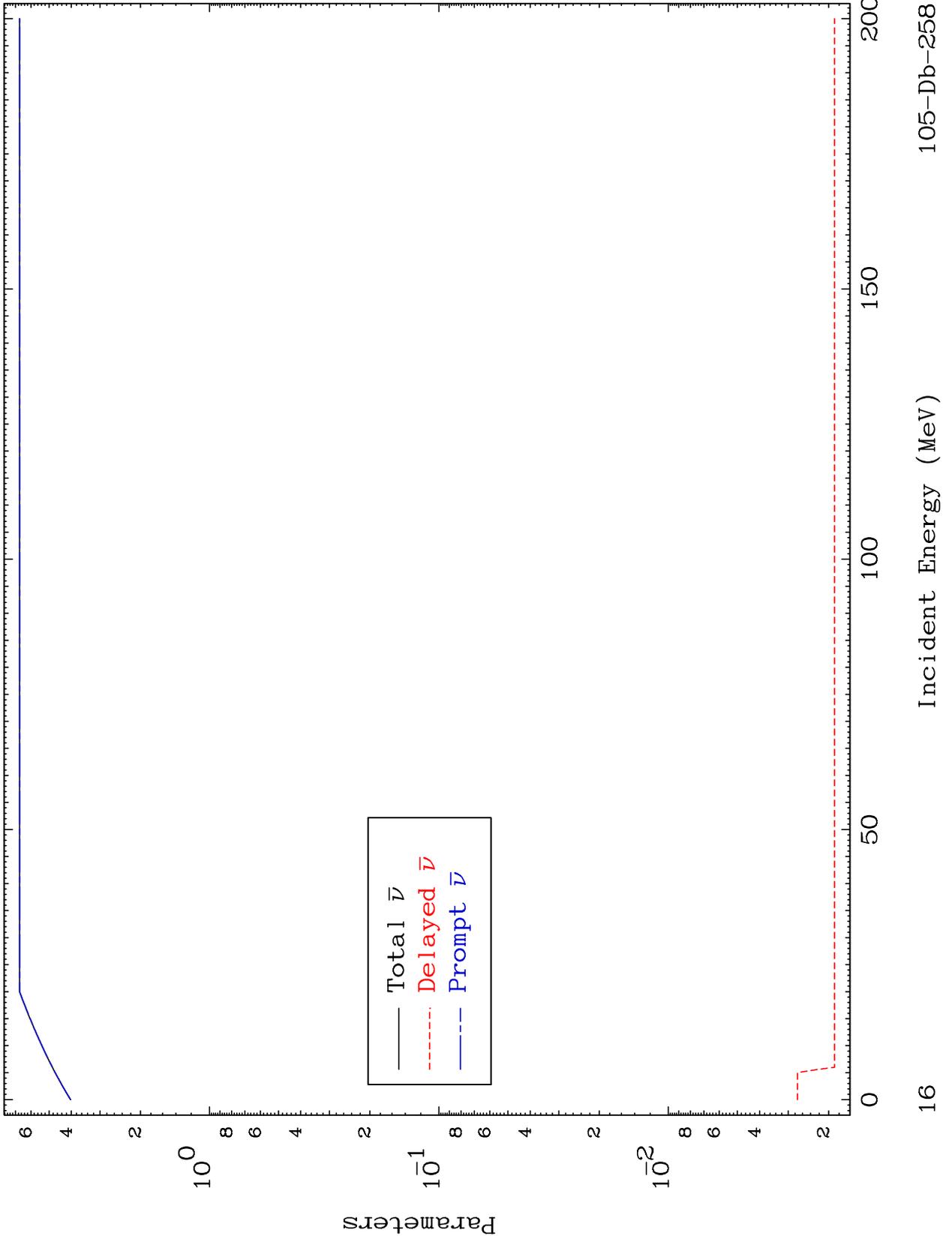
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Energy Release
Parameters

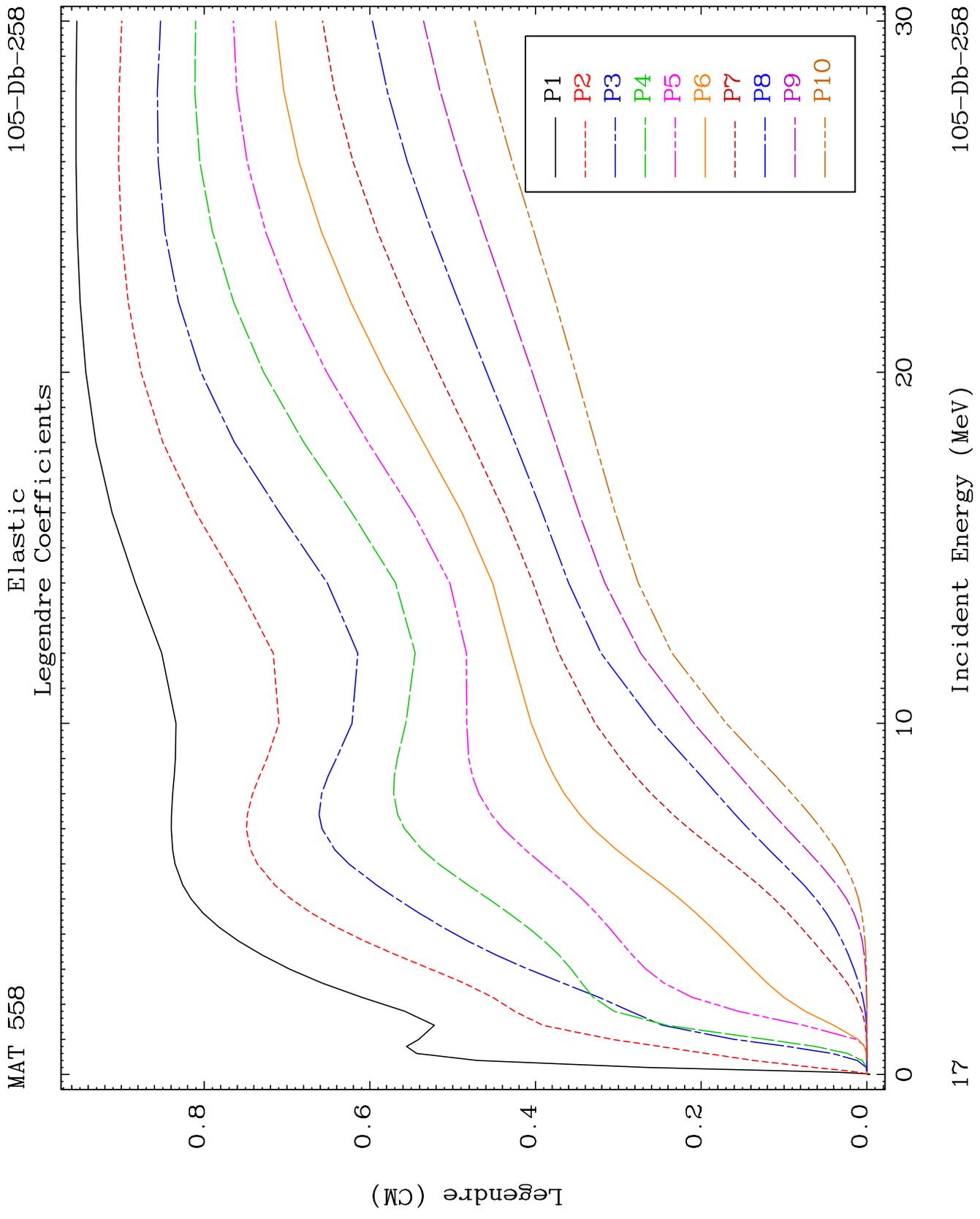
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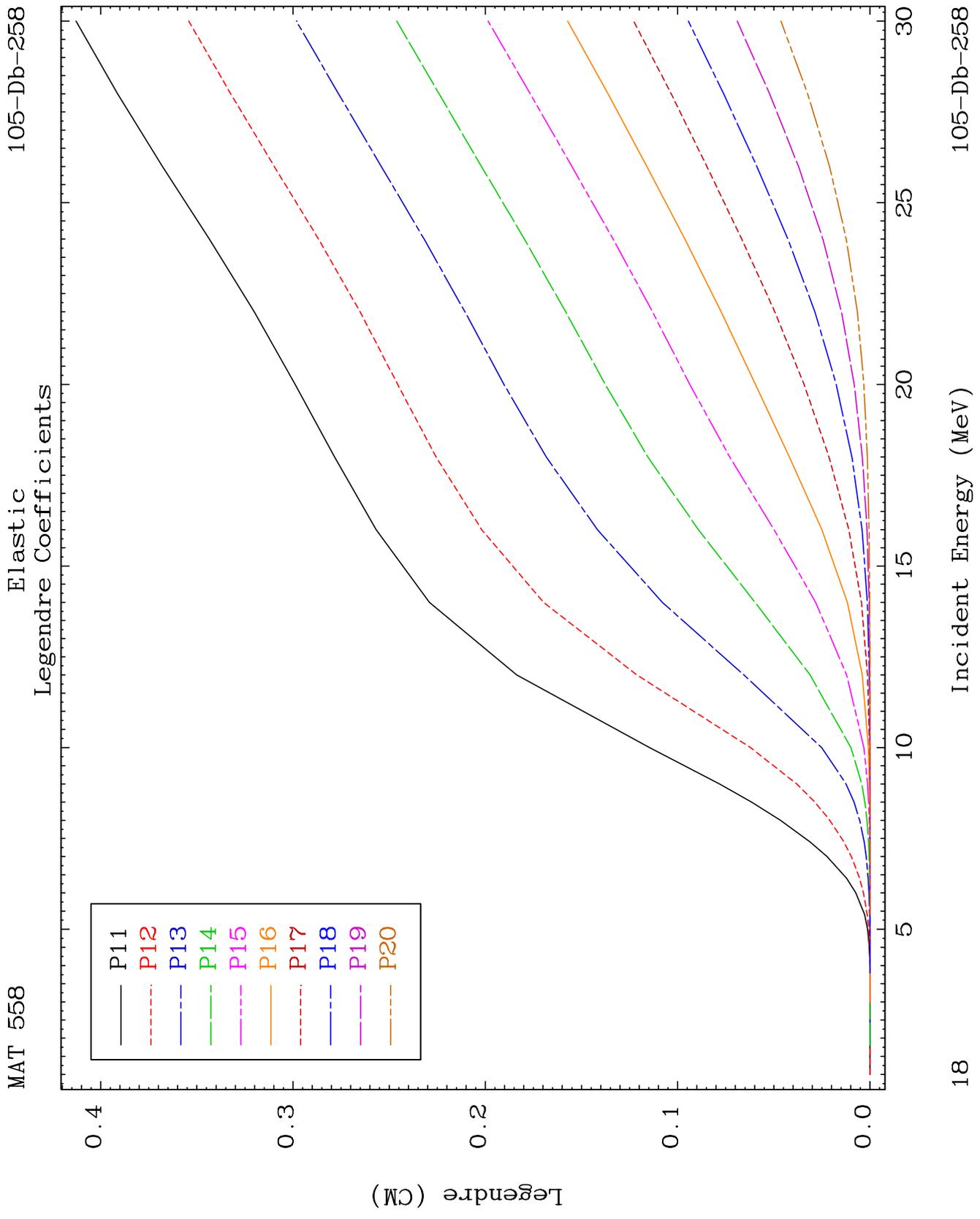


16

Incident Energy (MeV)

105-Db-258

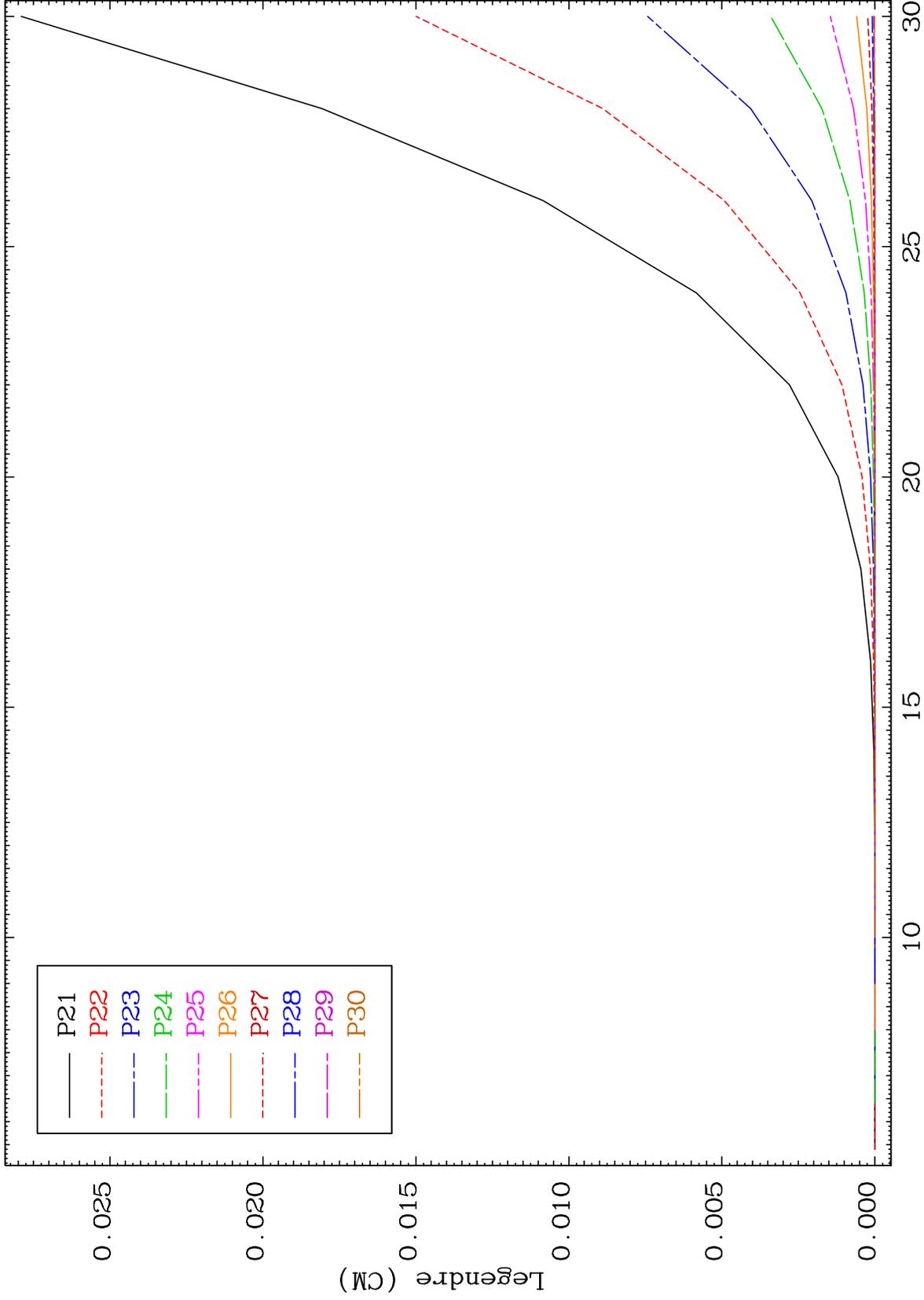




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Elastic Legendre Coefficients

105-Db-258



19

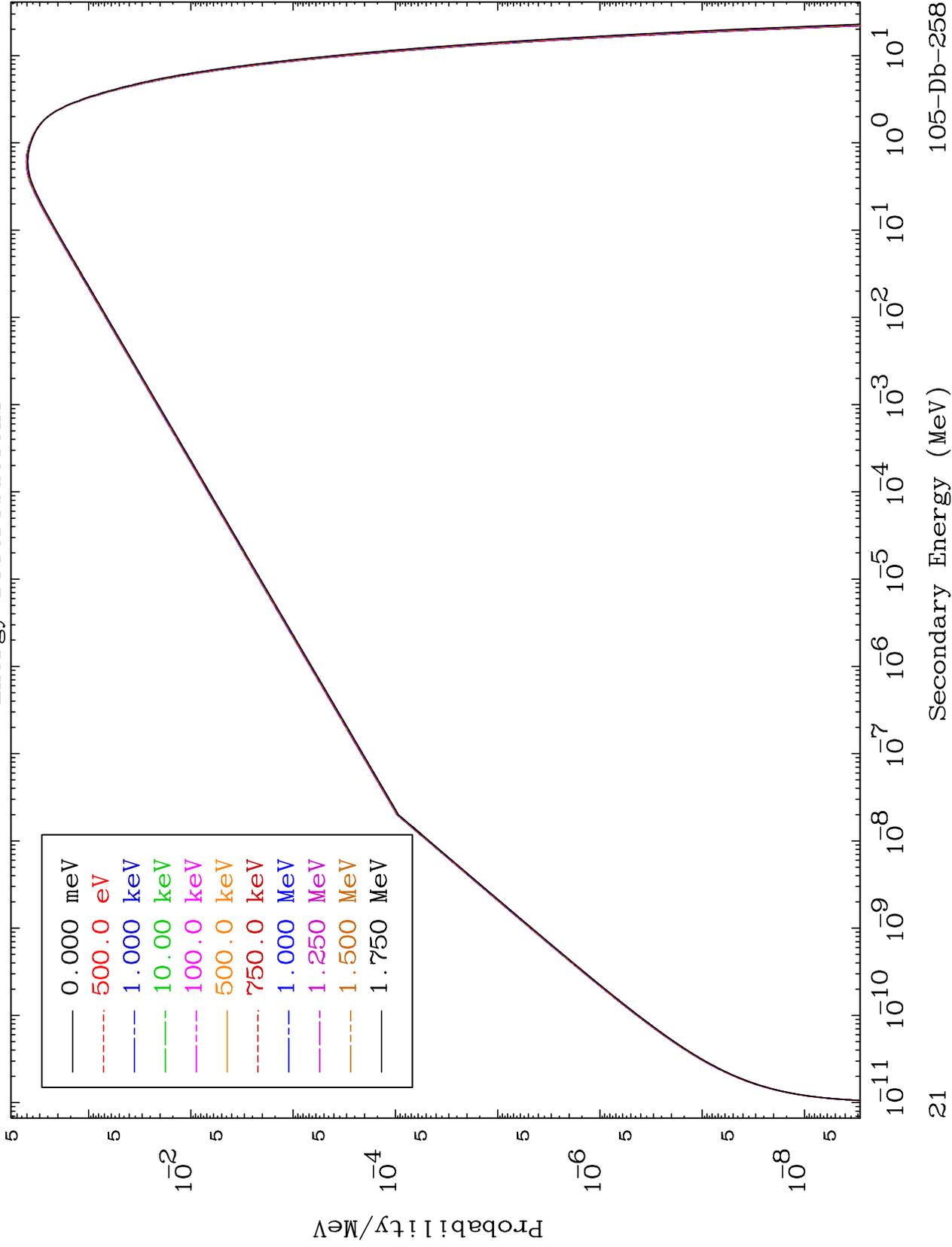
Incident Energy (MeV)

105-Db-258

MAT 558

Fission Energy Distributions

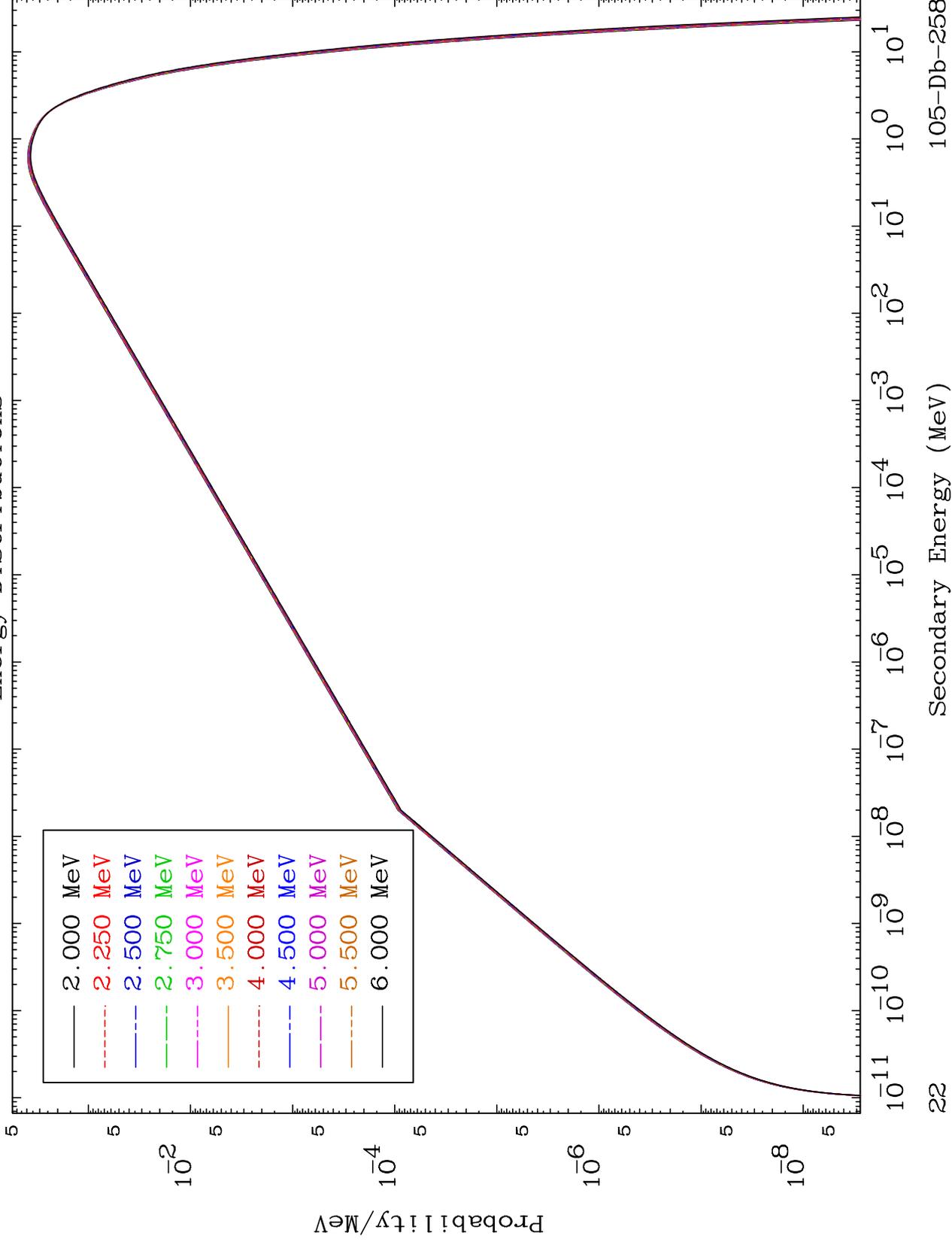
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Fission Energy Distributions

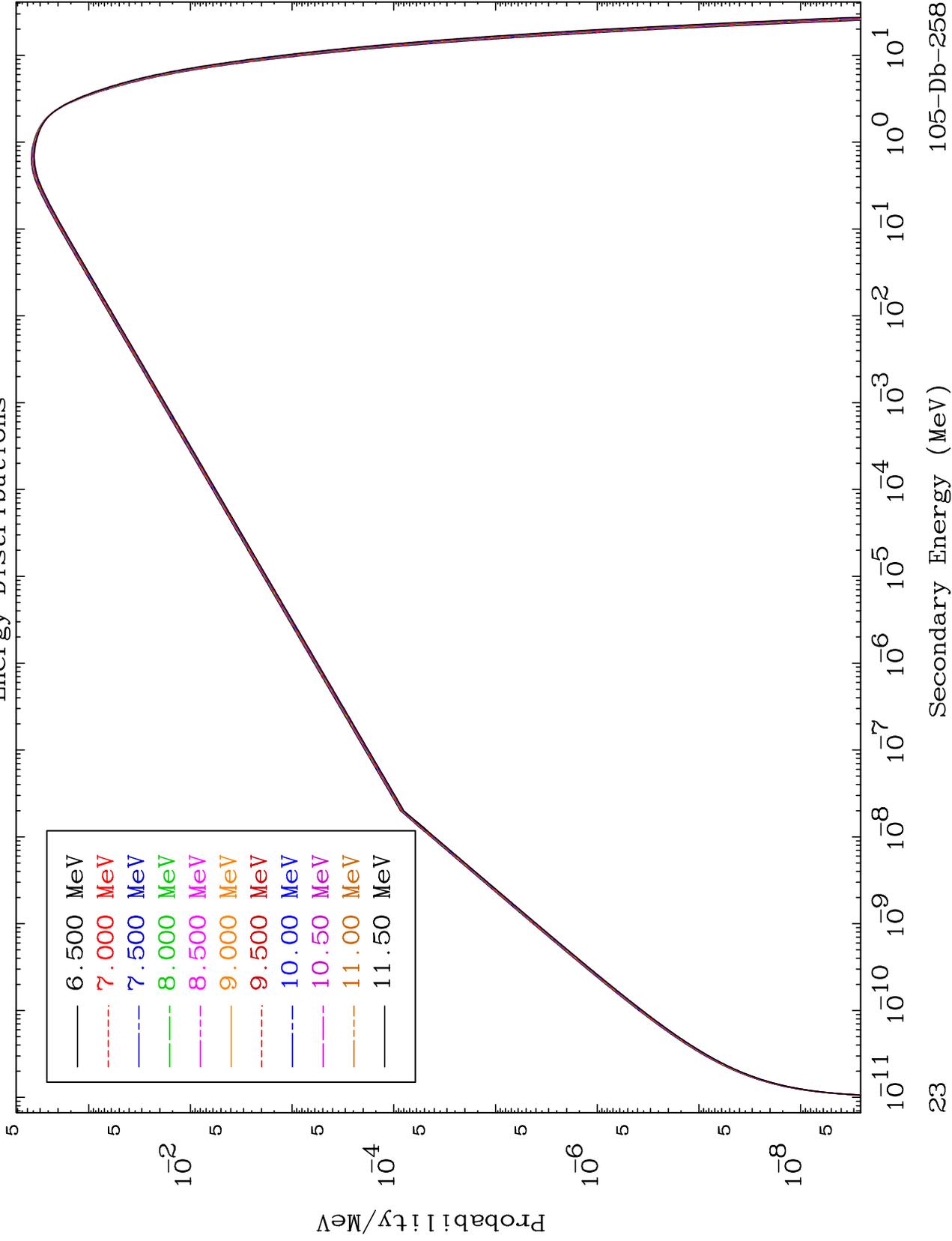
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Fission Energy Distributions

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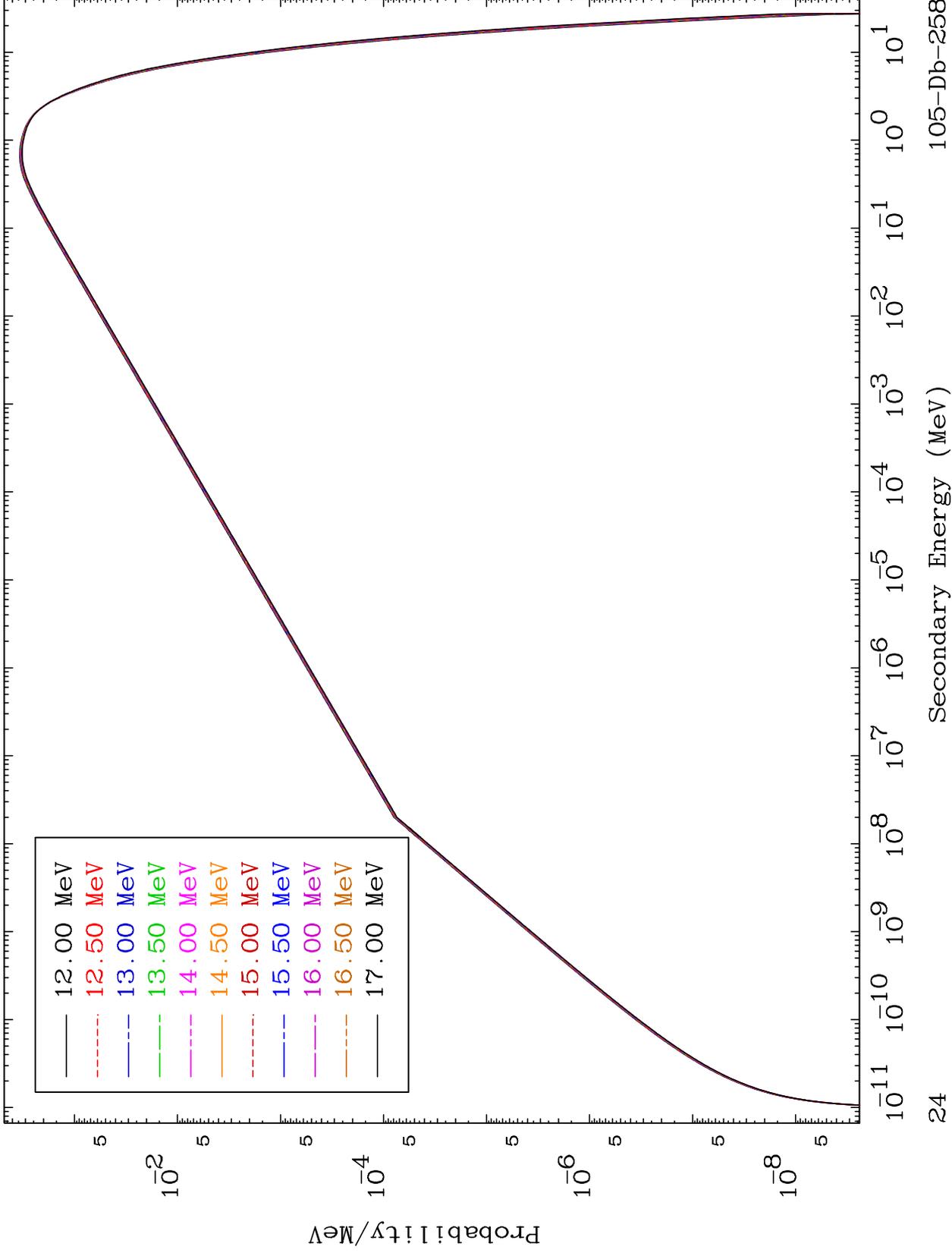
23

105-Db-258

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Fission Energy Distributions

105-Db-258



105-Db-258

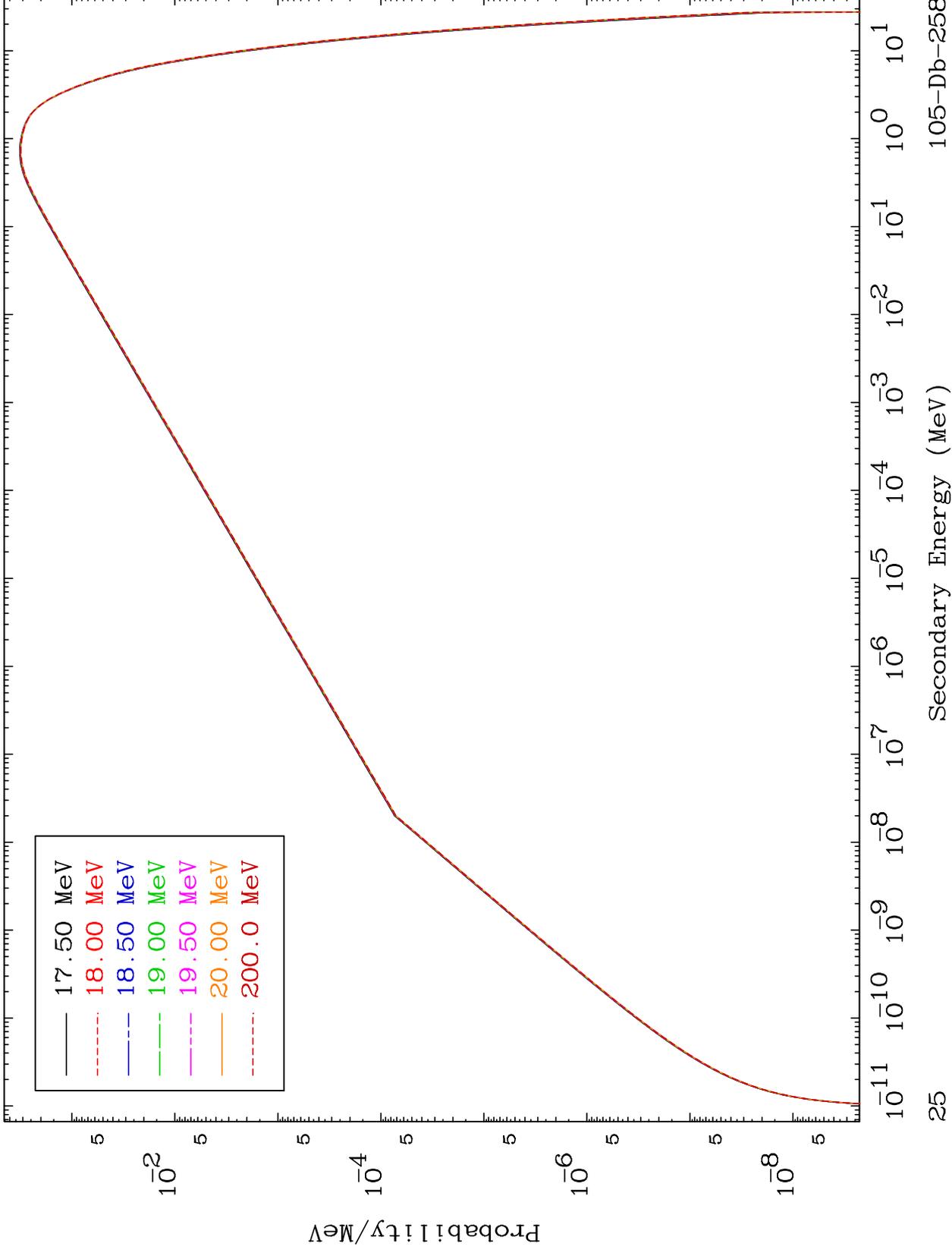
Secondary Energy (MeV)

24

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Fission Energy Distributions

105-Db-258

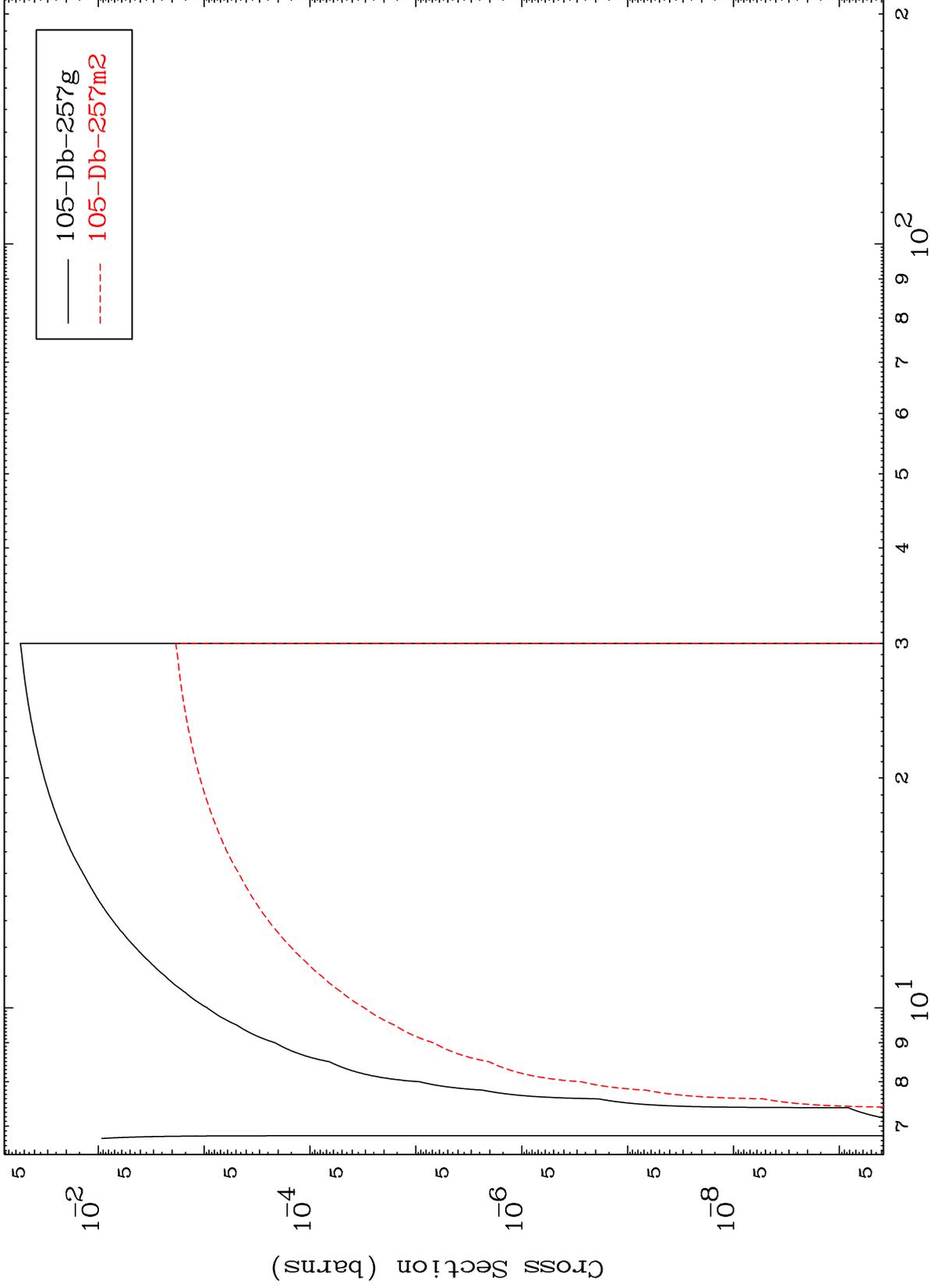


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(n,2n)

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Radionuclide Production Cross Section



26

Incident Energy (MeV)

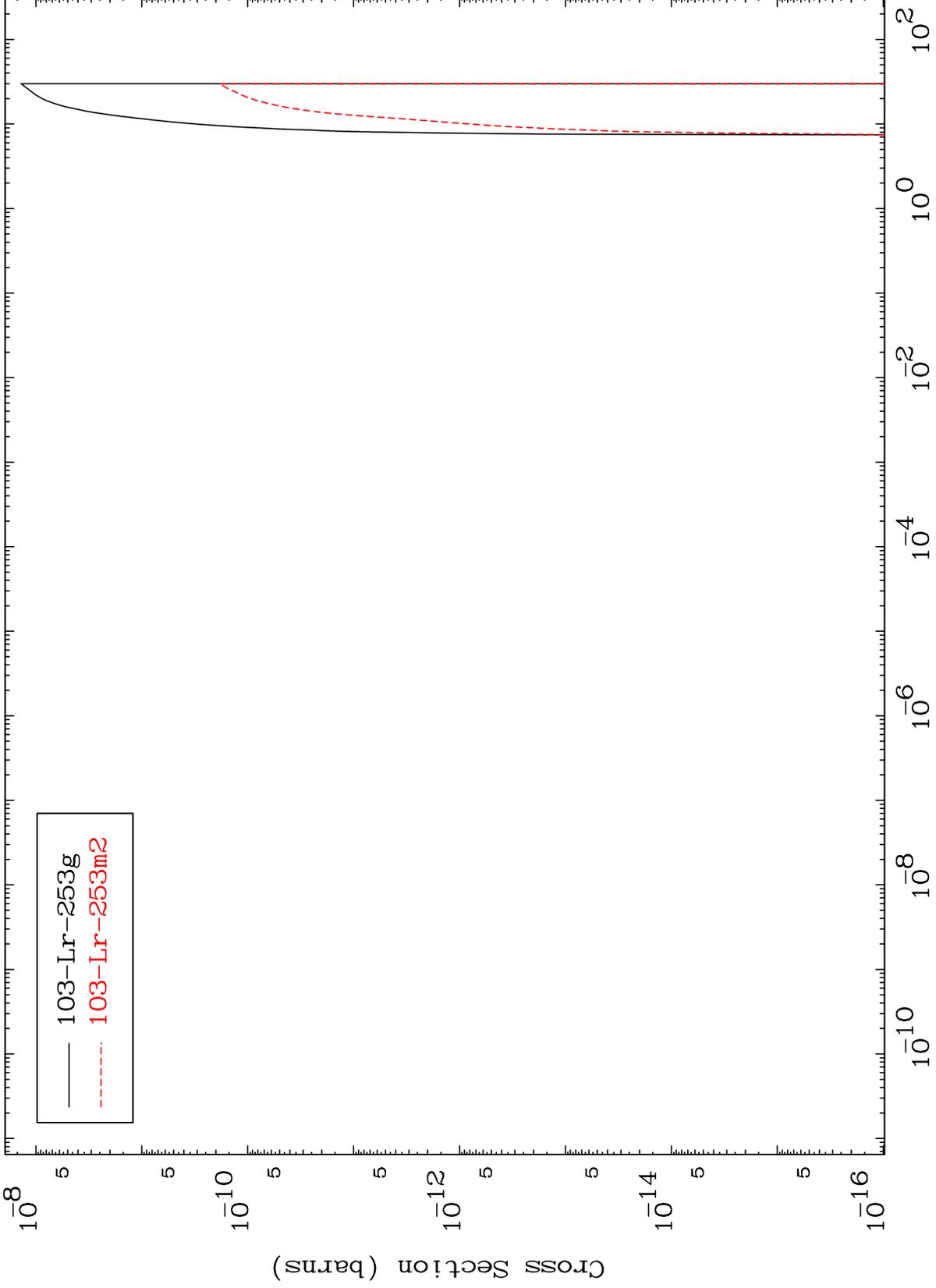
105-Db-258

MAT 558

(n,2n) α

105-Db-258

Radionuclide Production Cross Section

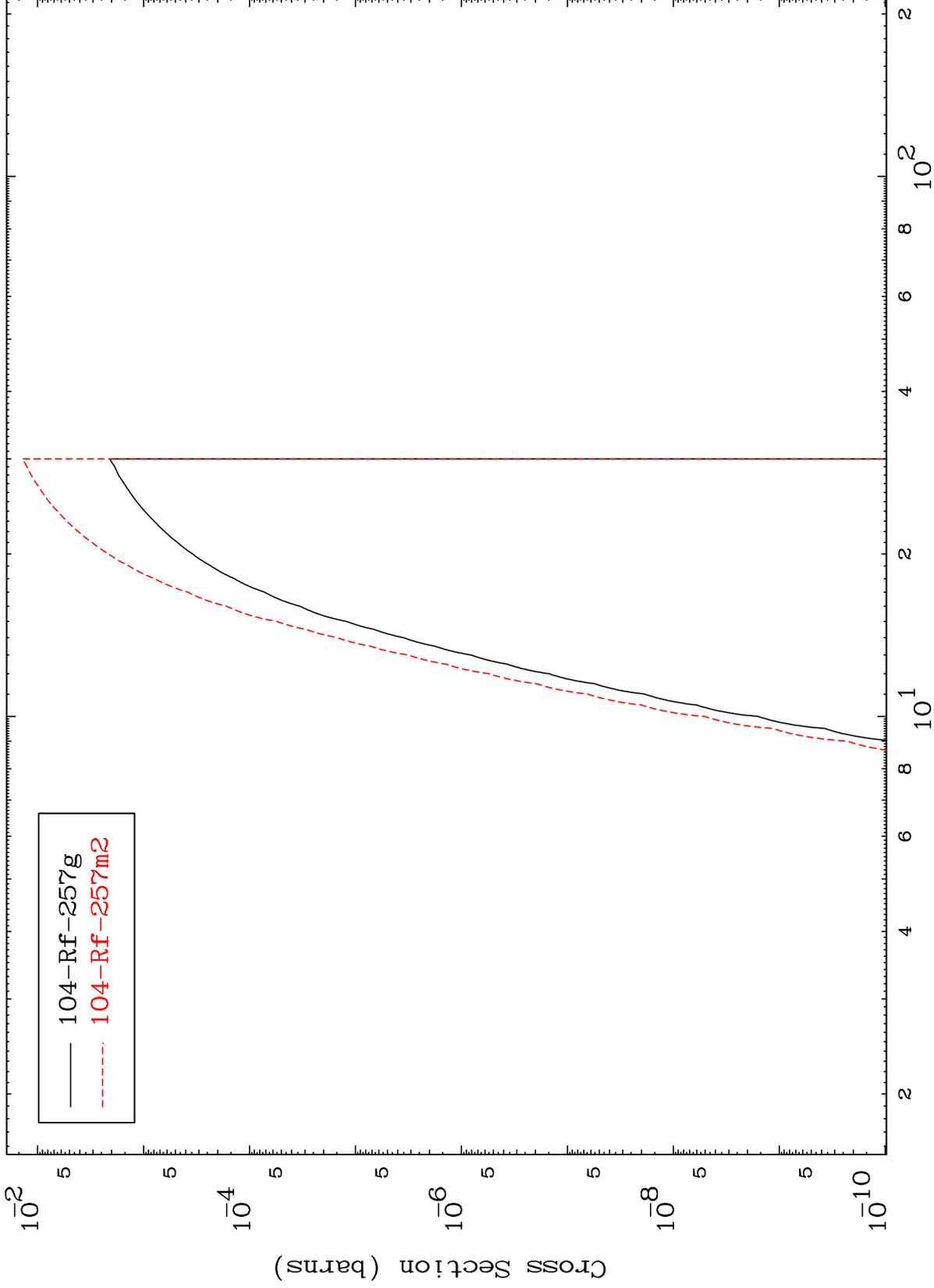


MAT 558

(n,n') p

105-Db-258

Radionuclide Production Cross Section



28

Incident Energy (MeV)

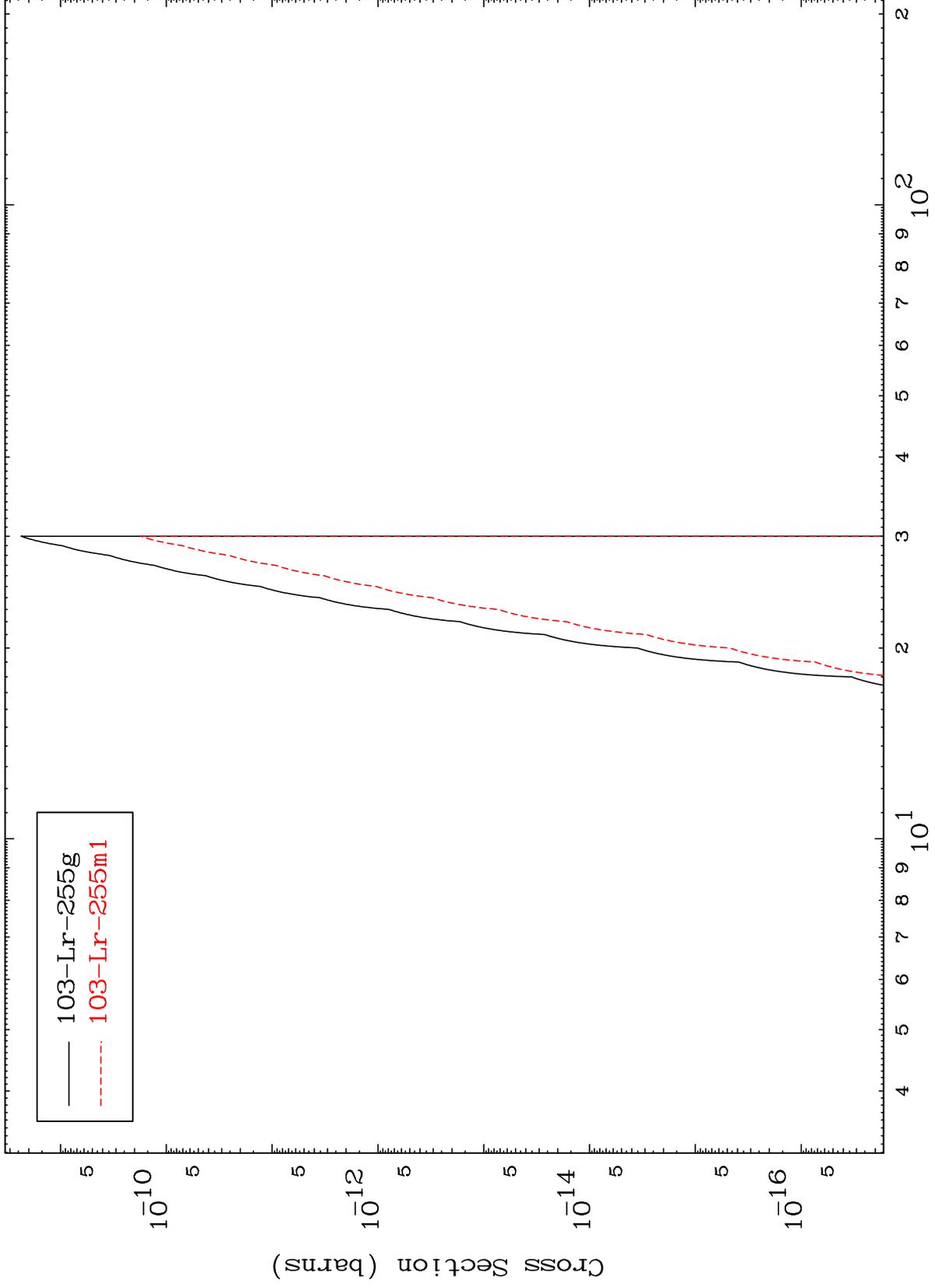
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(n,n') He-3

105-Db-258

Radionuclide Production Cross Section



29

Incident Energy (MeV)

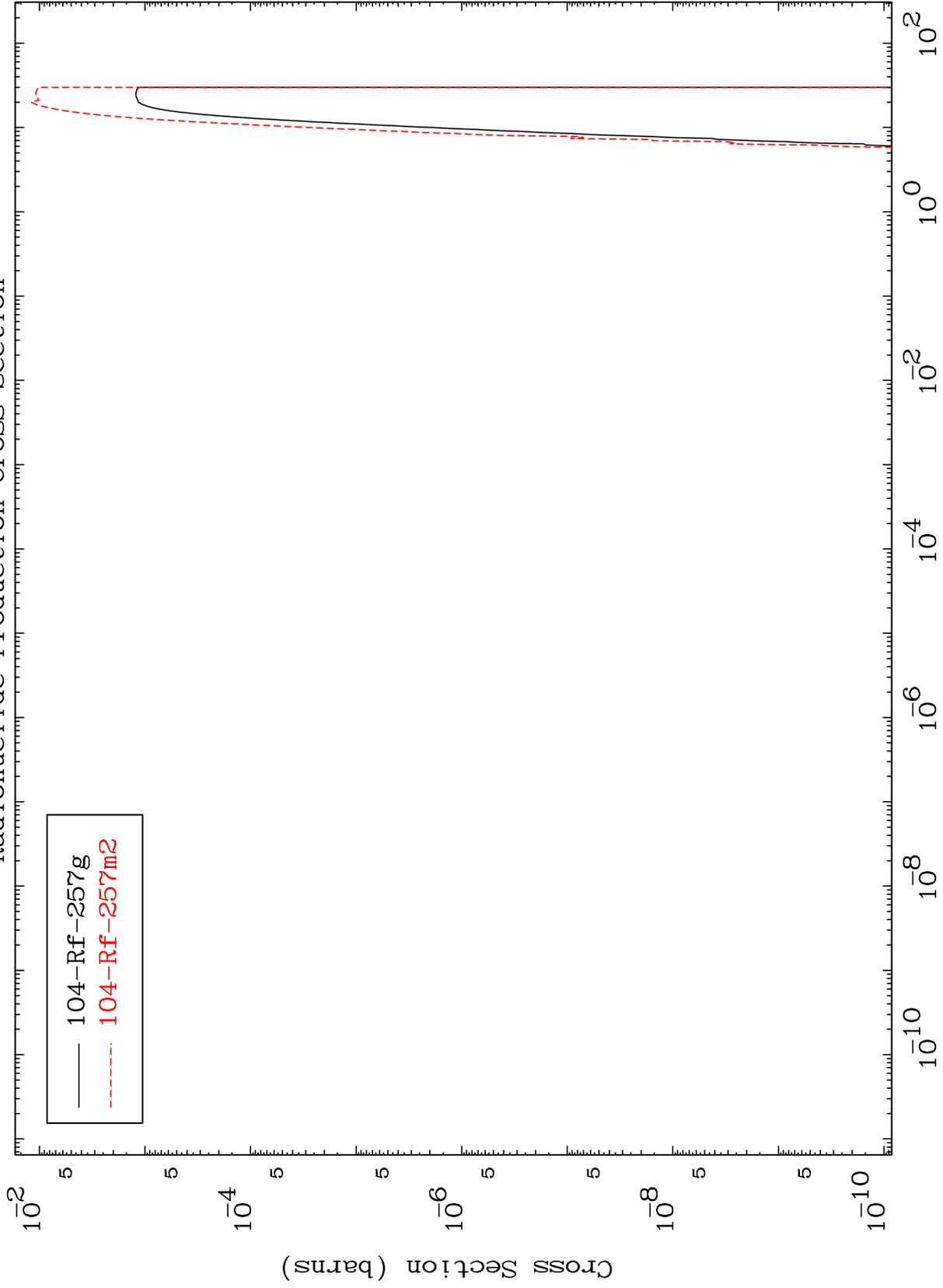
105-Db-258

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(n,d)

105-Db-258

Radionuclide Production Cross Section



30

Incident Energy (MeV)

105-Db-258

