

Program EVALPLOT
(Version 2021-1)

by

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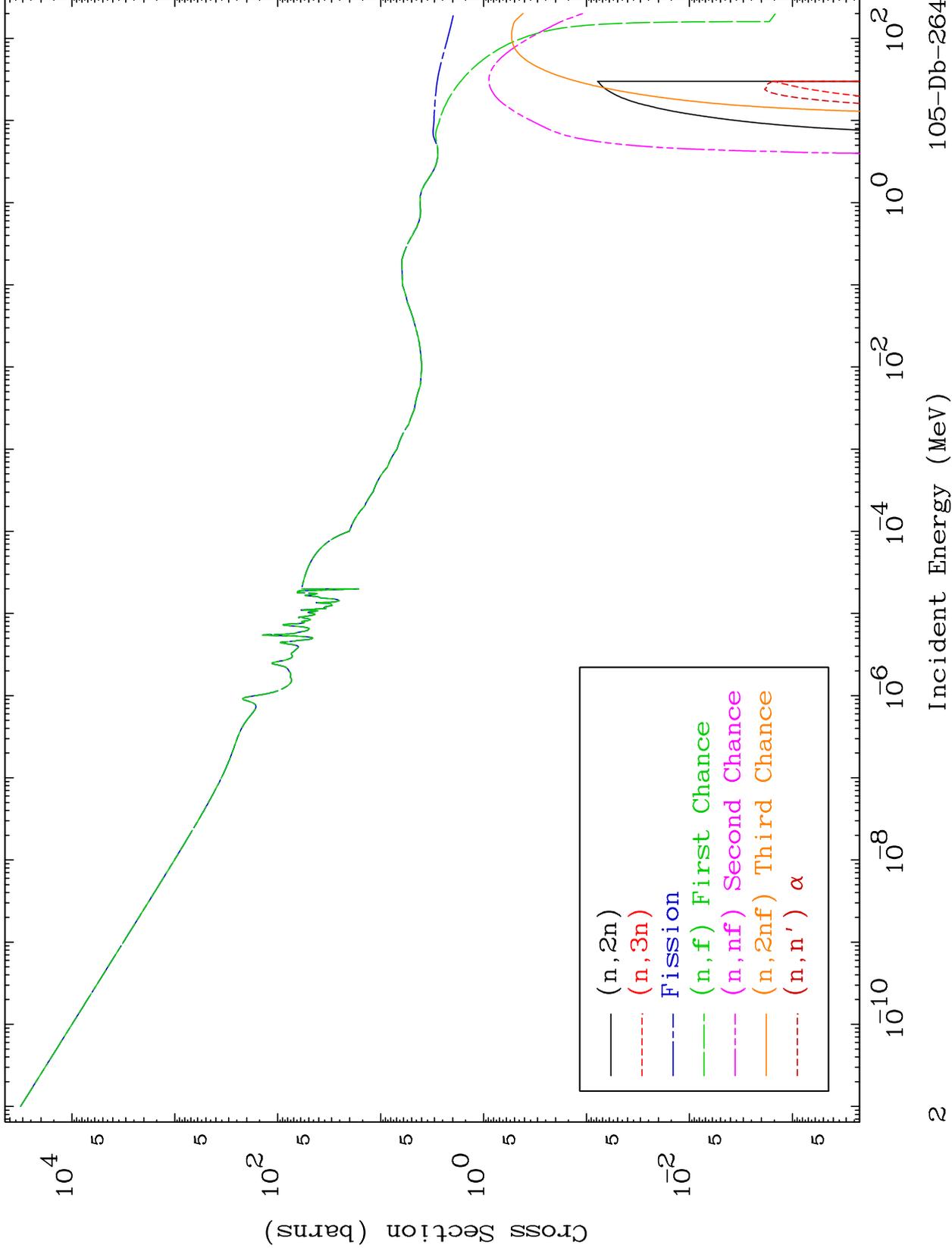
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Press Mouse Button to Start

MAT 564

Neutron Absorption
293 Kelvin Cross Sections

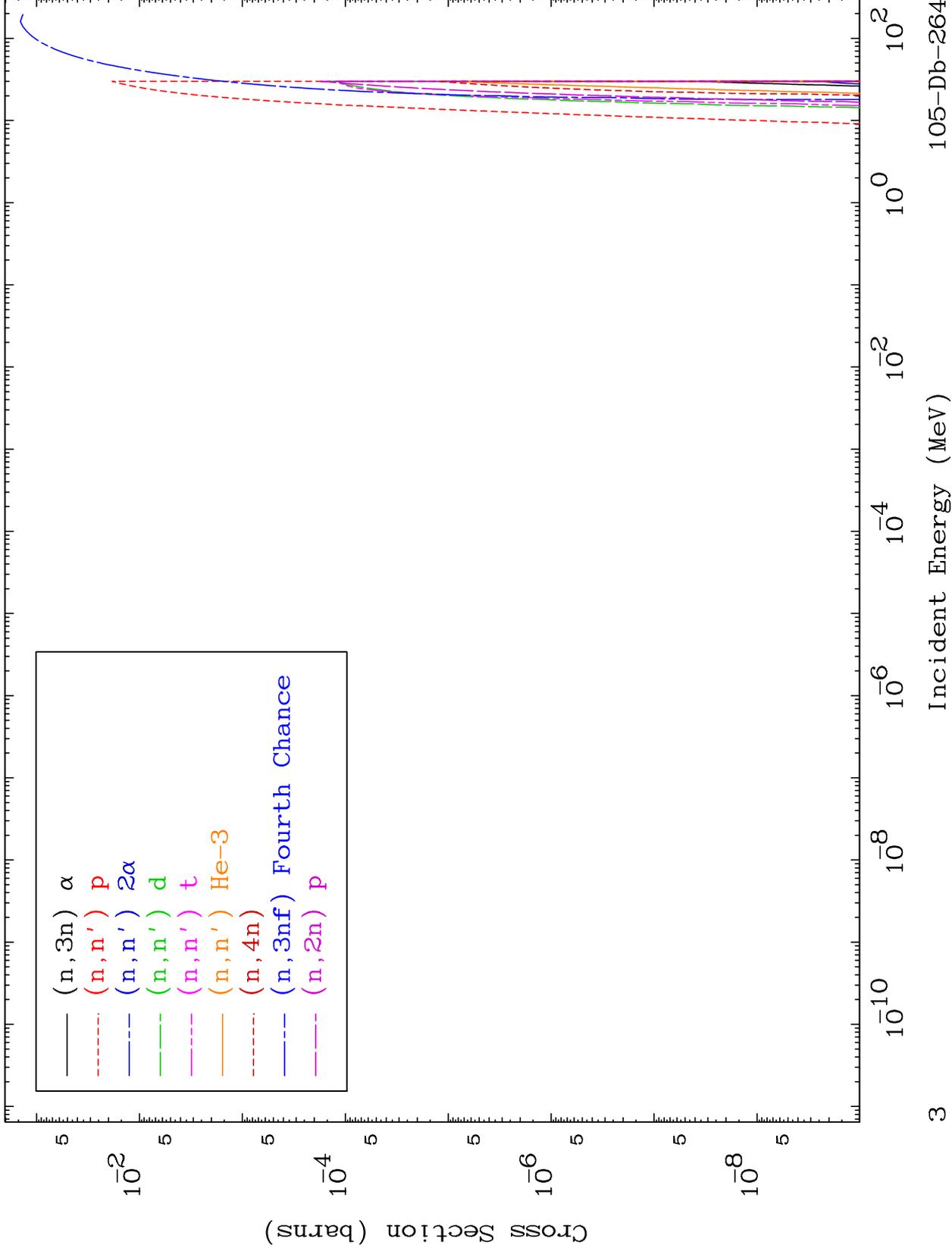
105-Db-264



MAT 564

Neutron Absorption
293 Kelvin Cross Sections

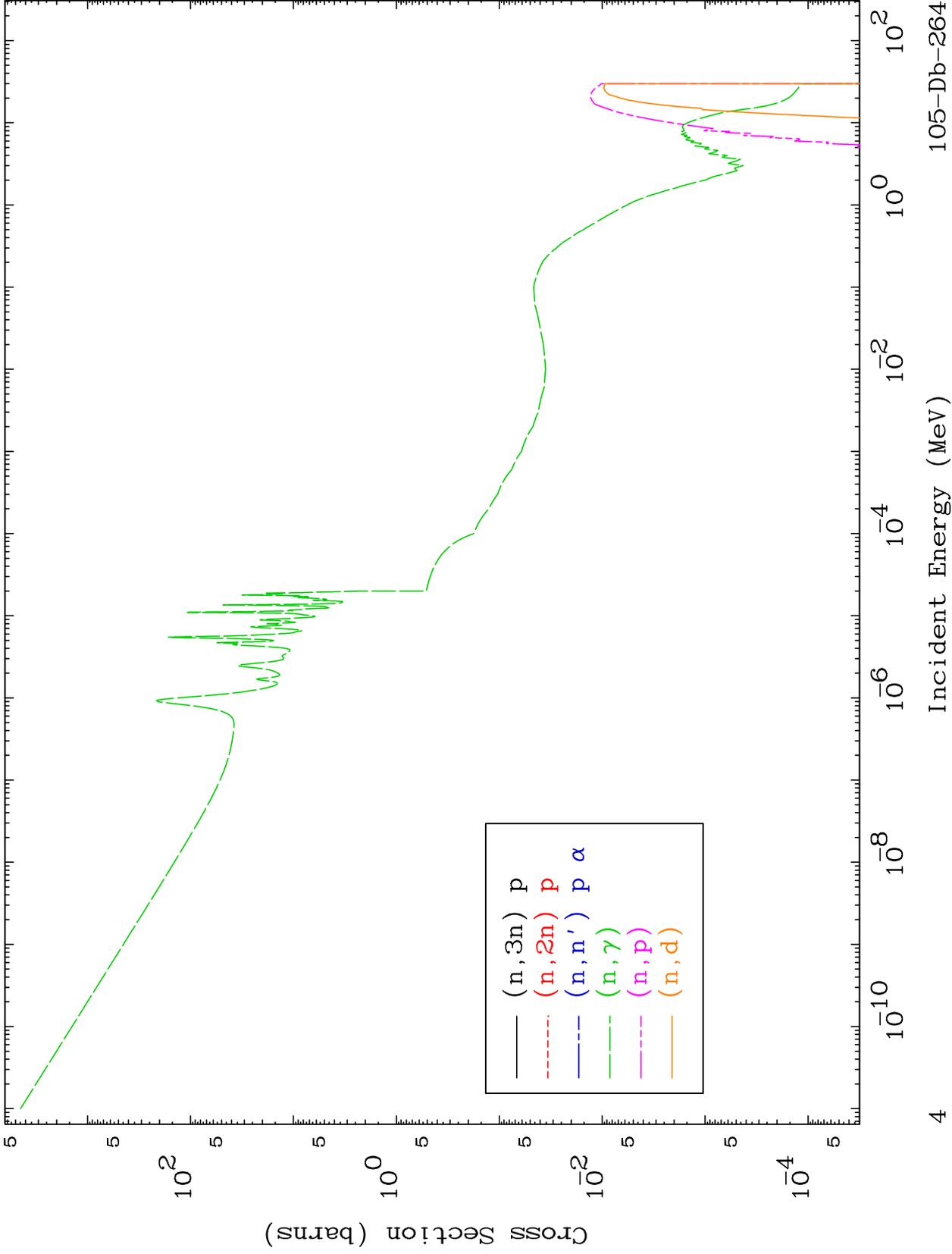
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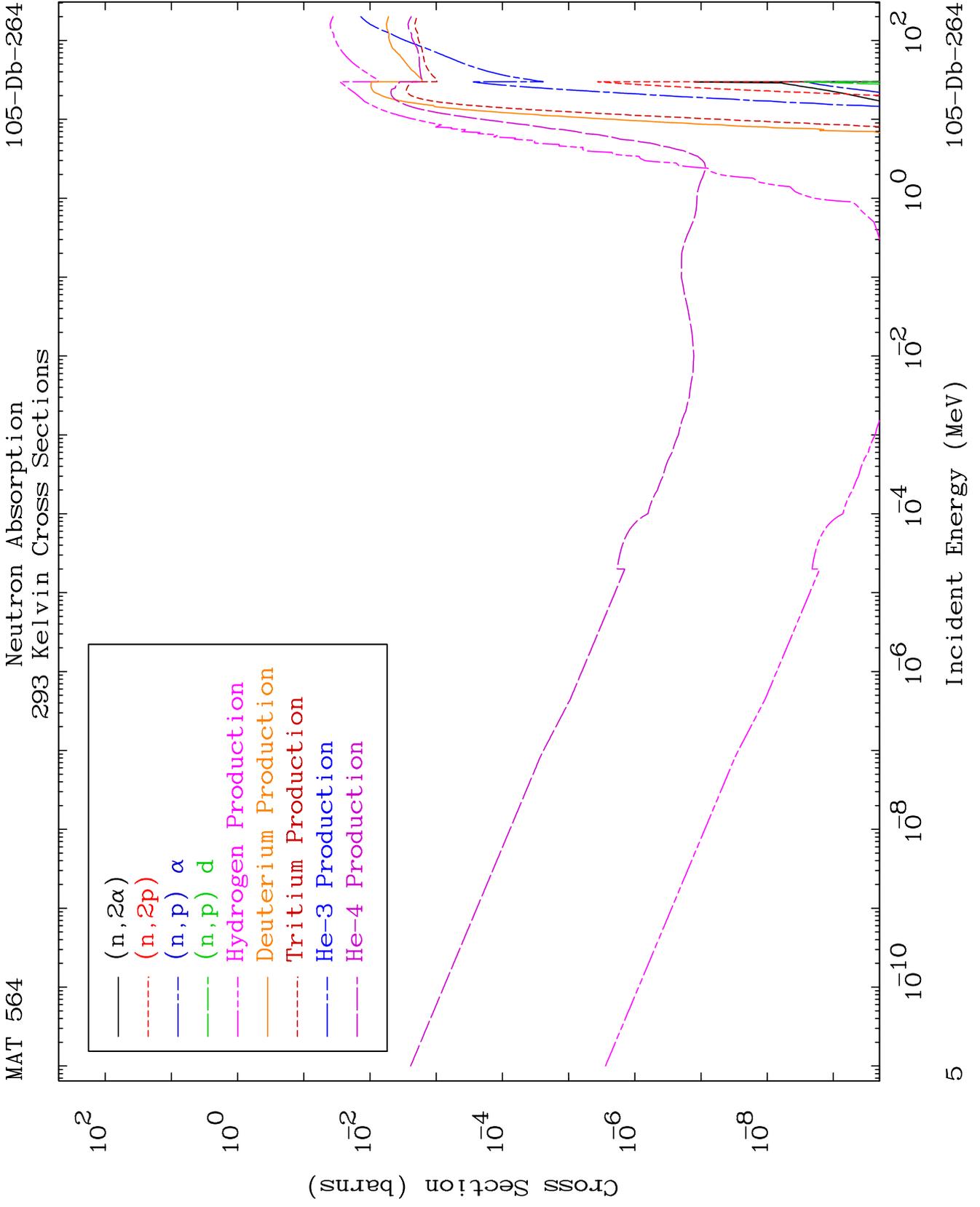


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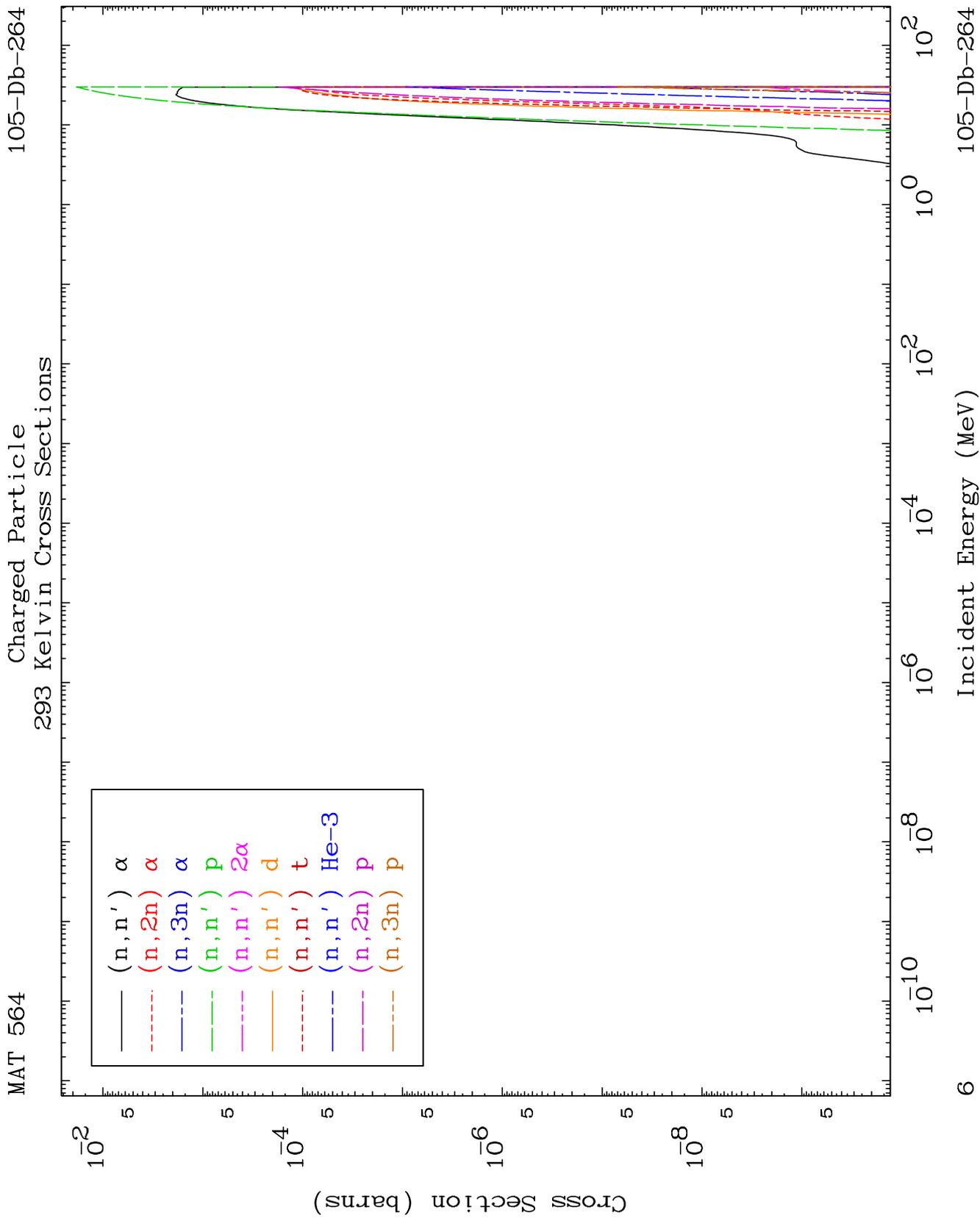
Neutron Absorption
293 Kelvin Cross Sections

105-Db-264





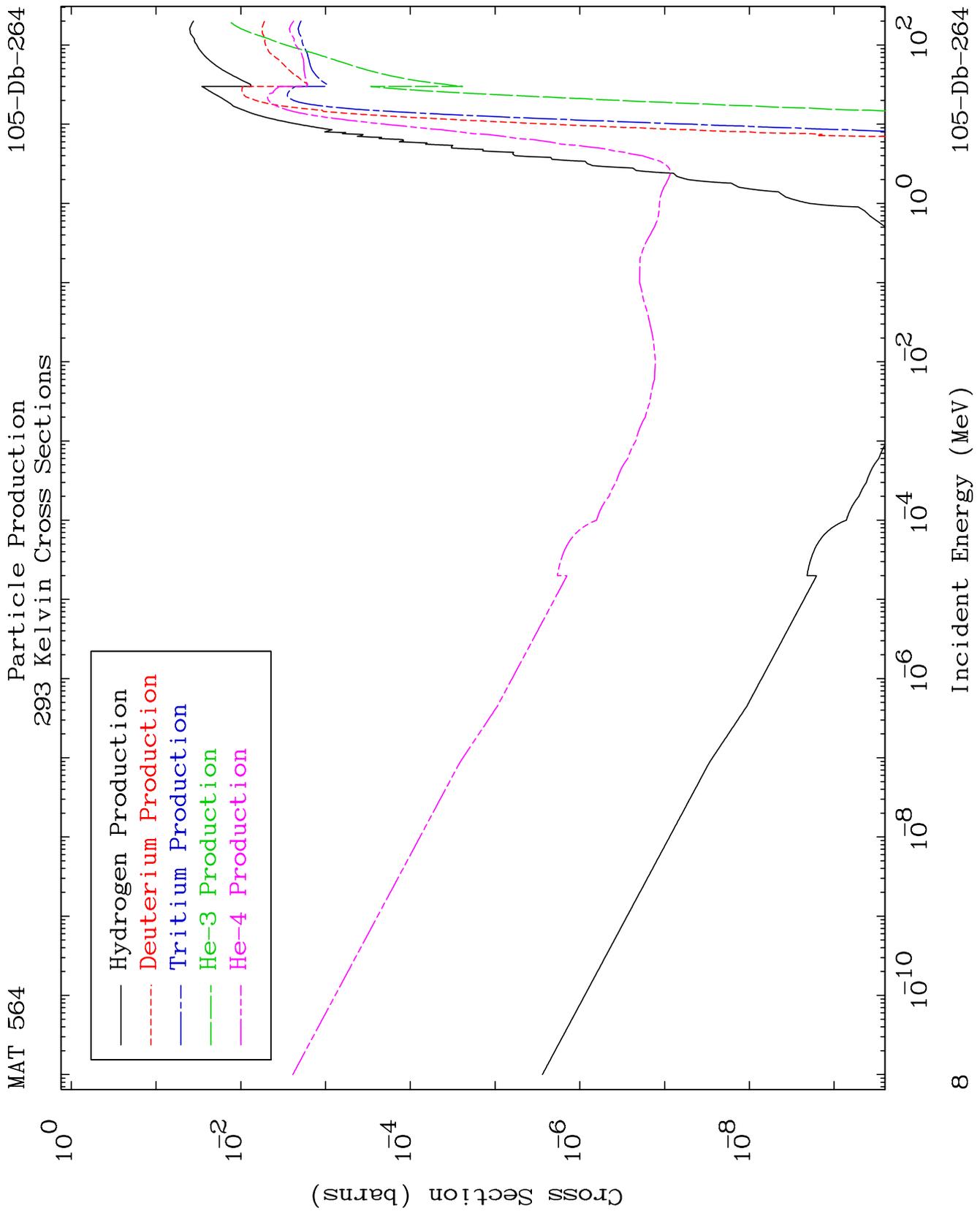
MAT 564



MAT 564

293 Kelvin Cross Sections

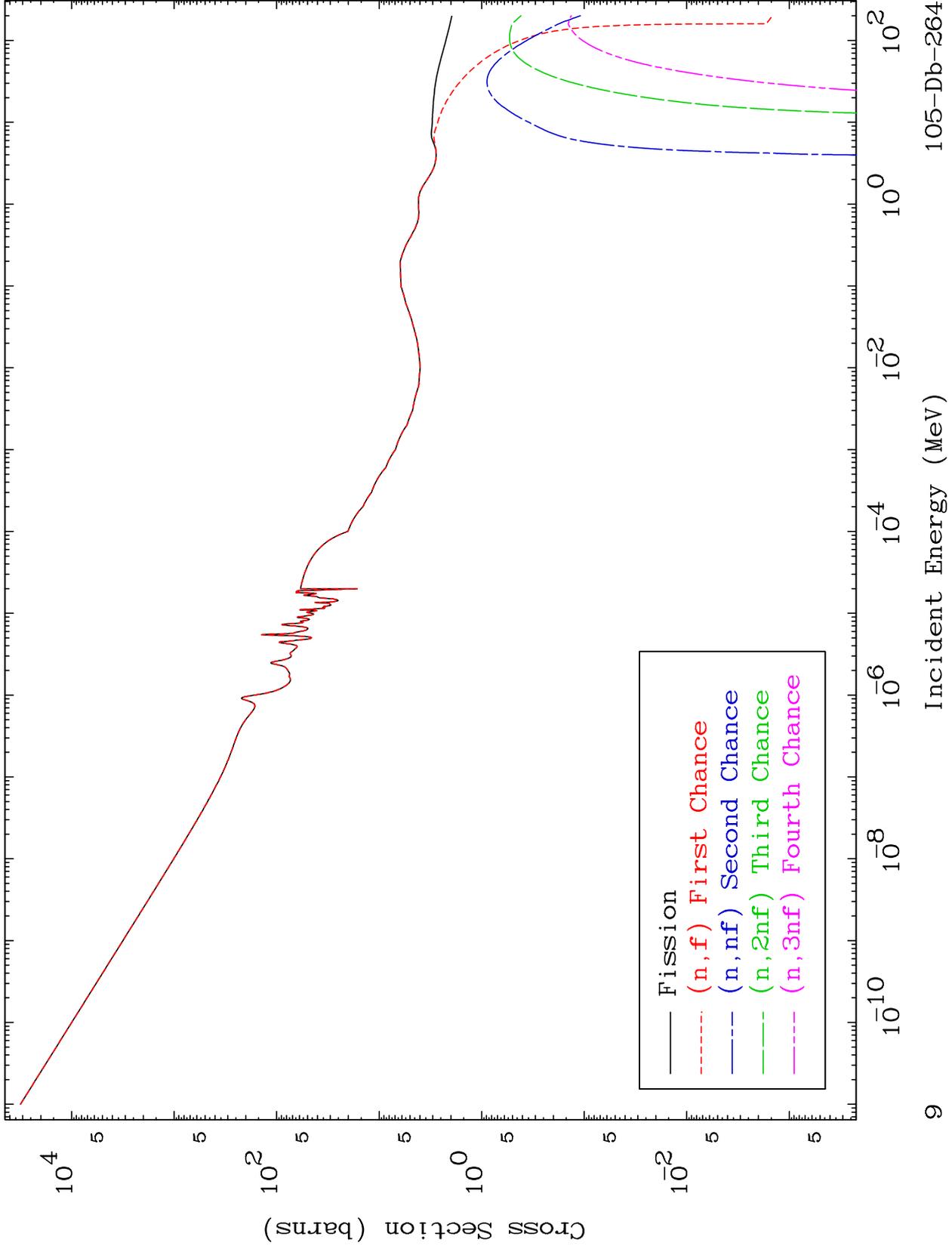
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MAT 564

Fission
293 Kelvin Cross Sections

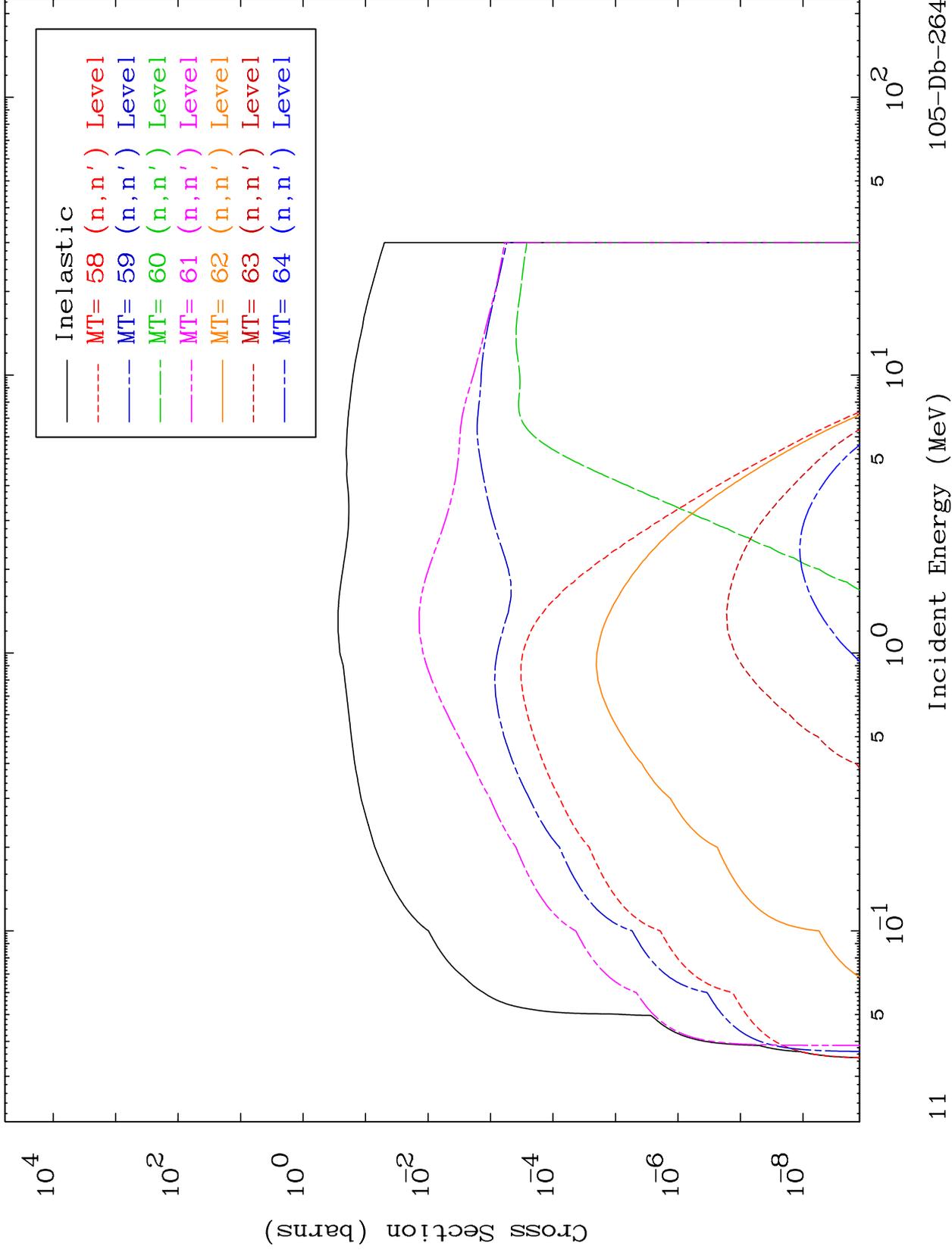
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MAT 564

(n,n') Levels
293 Kelvin Cross Sections

105-Db-264



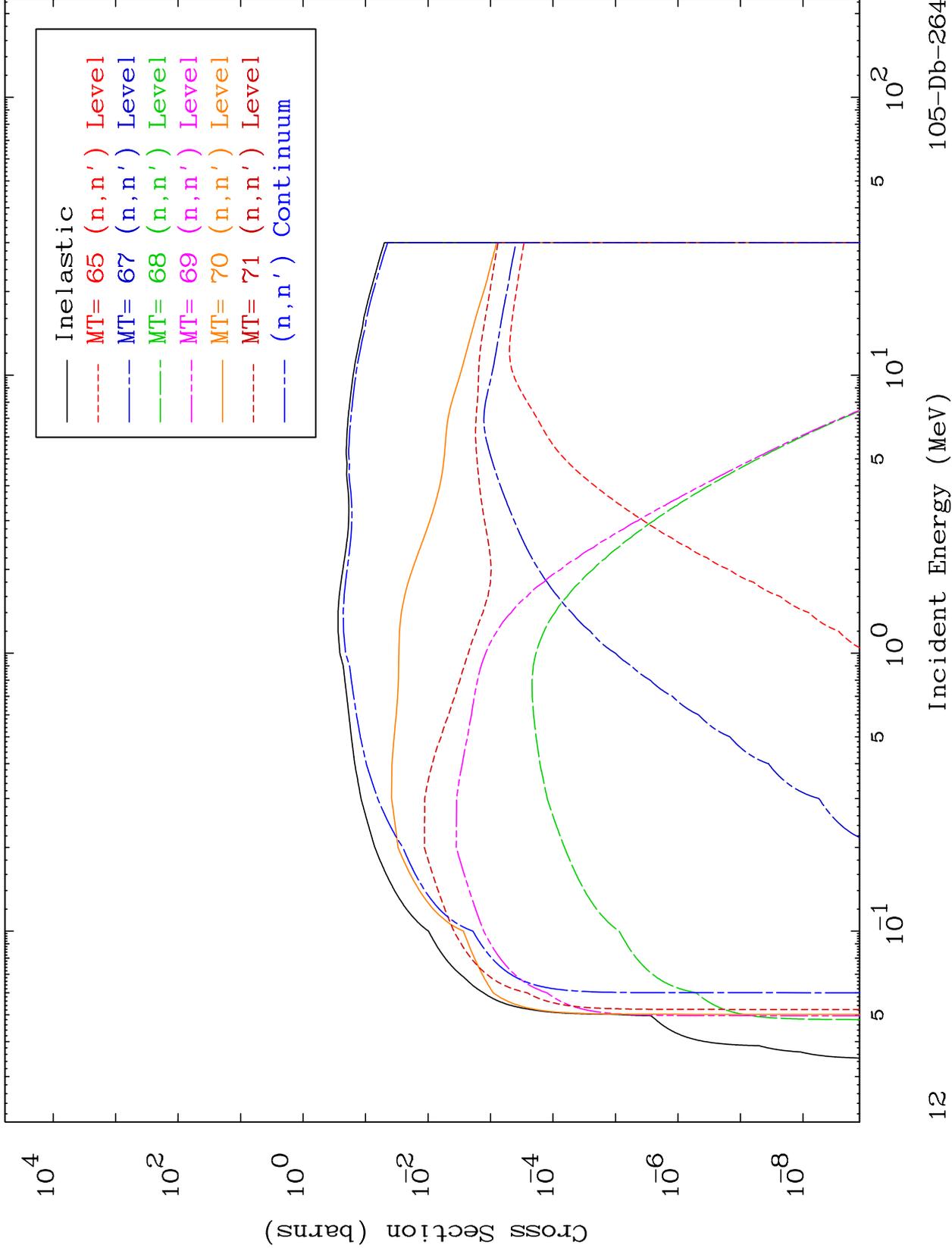
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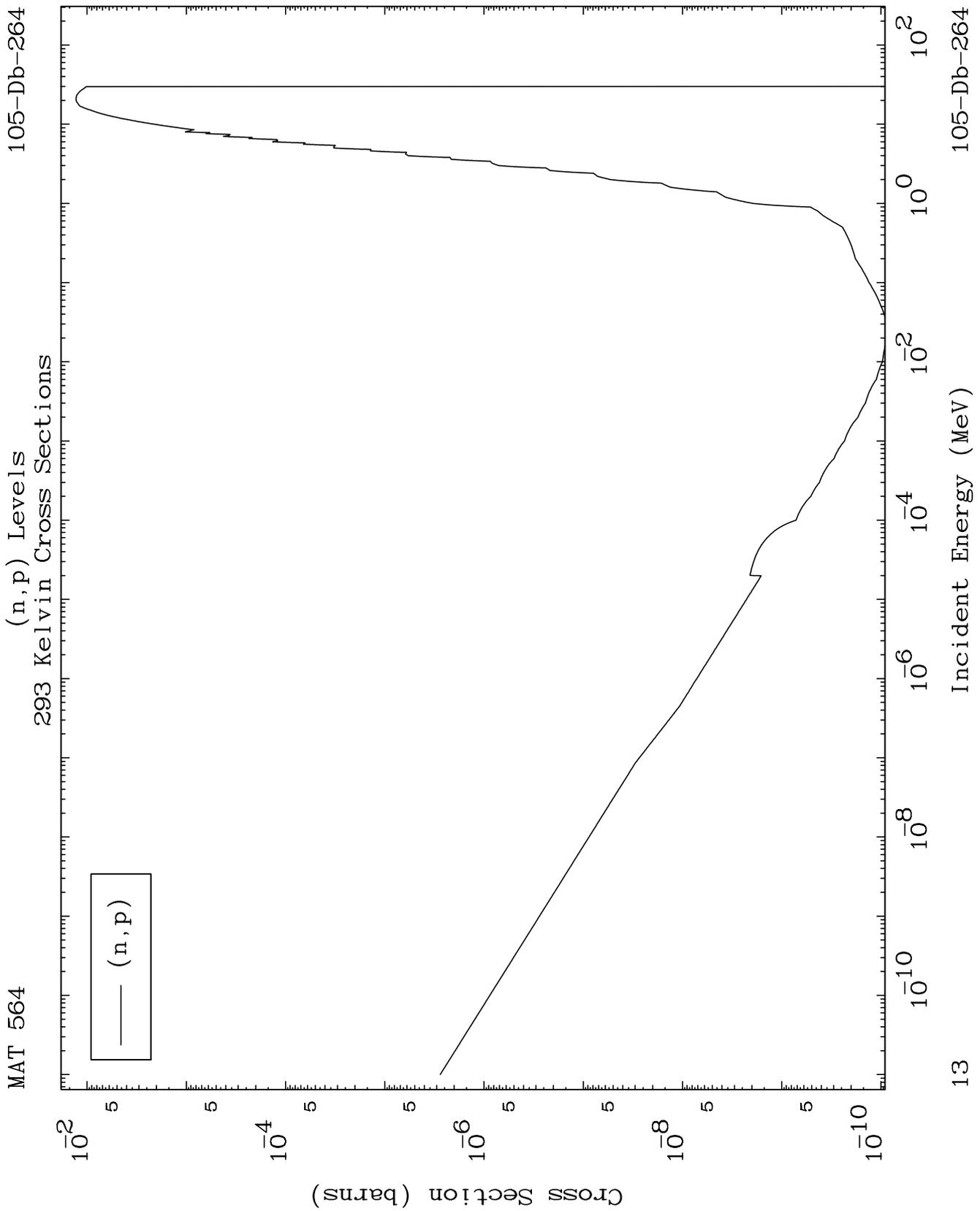
(n,n') Levels
293 Kelvin Cross Sections

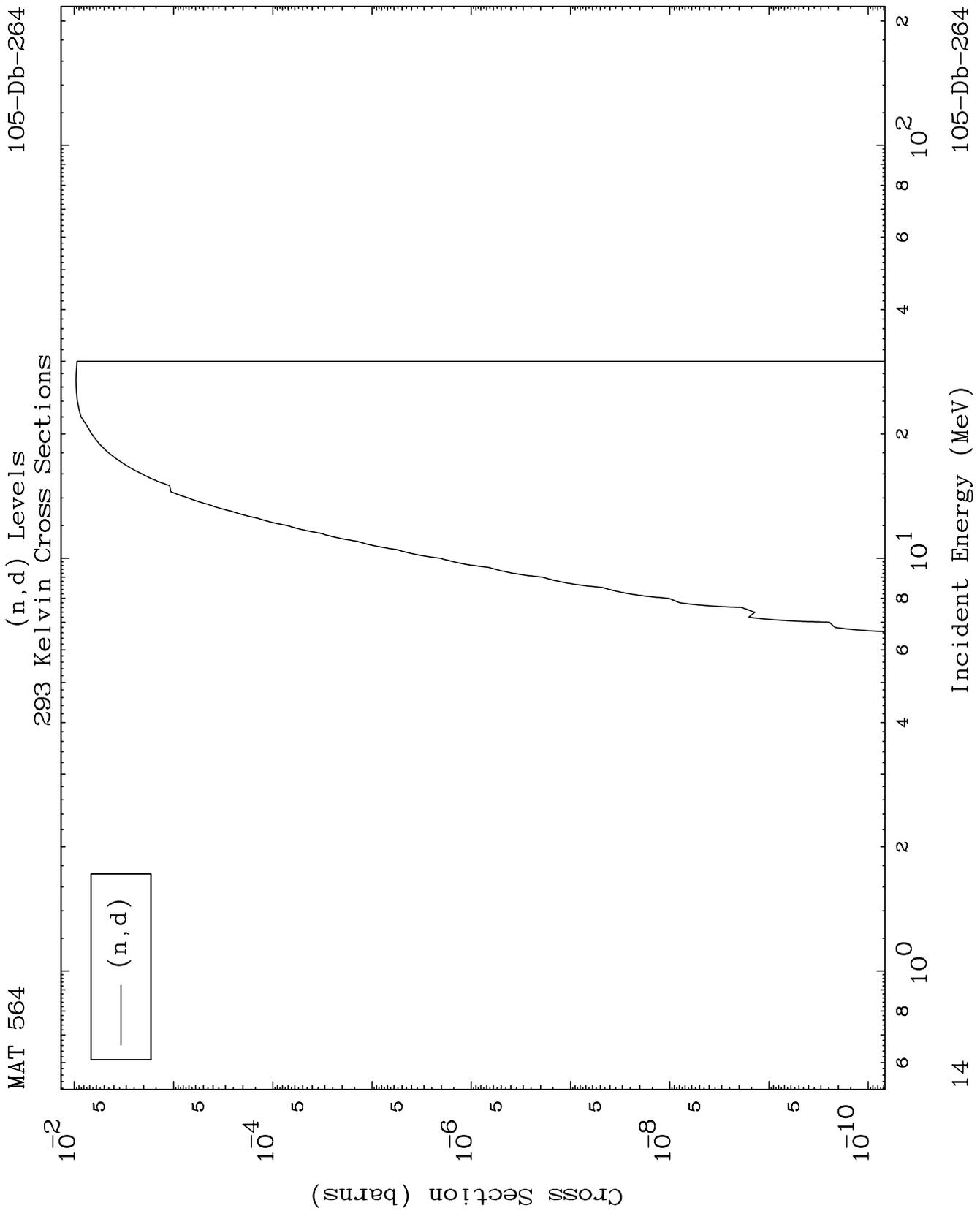
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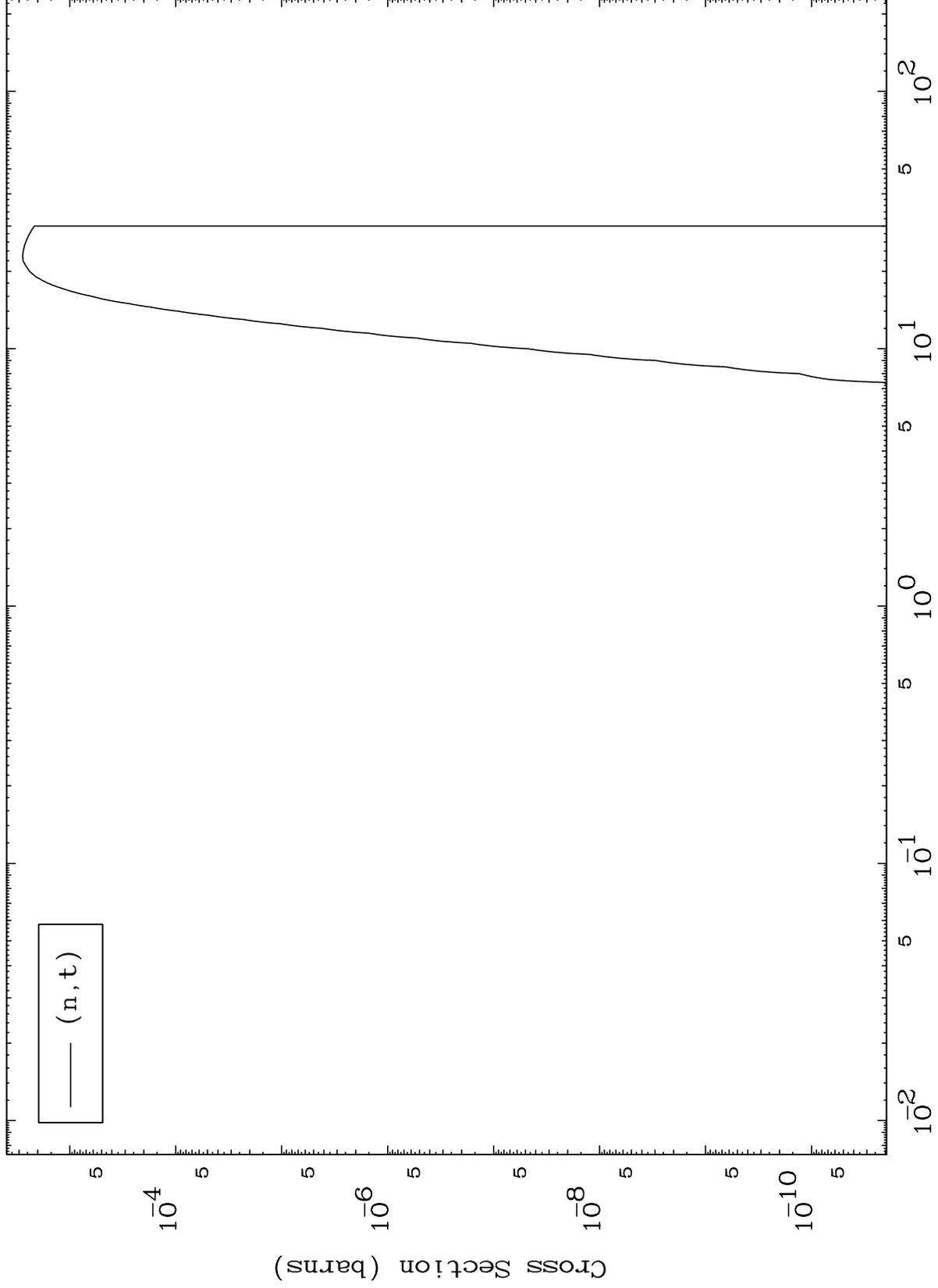




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(n,t) Levels
293 Kelvin Cross Sections

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105-Db-264

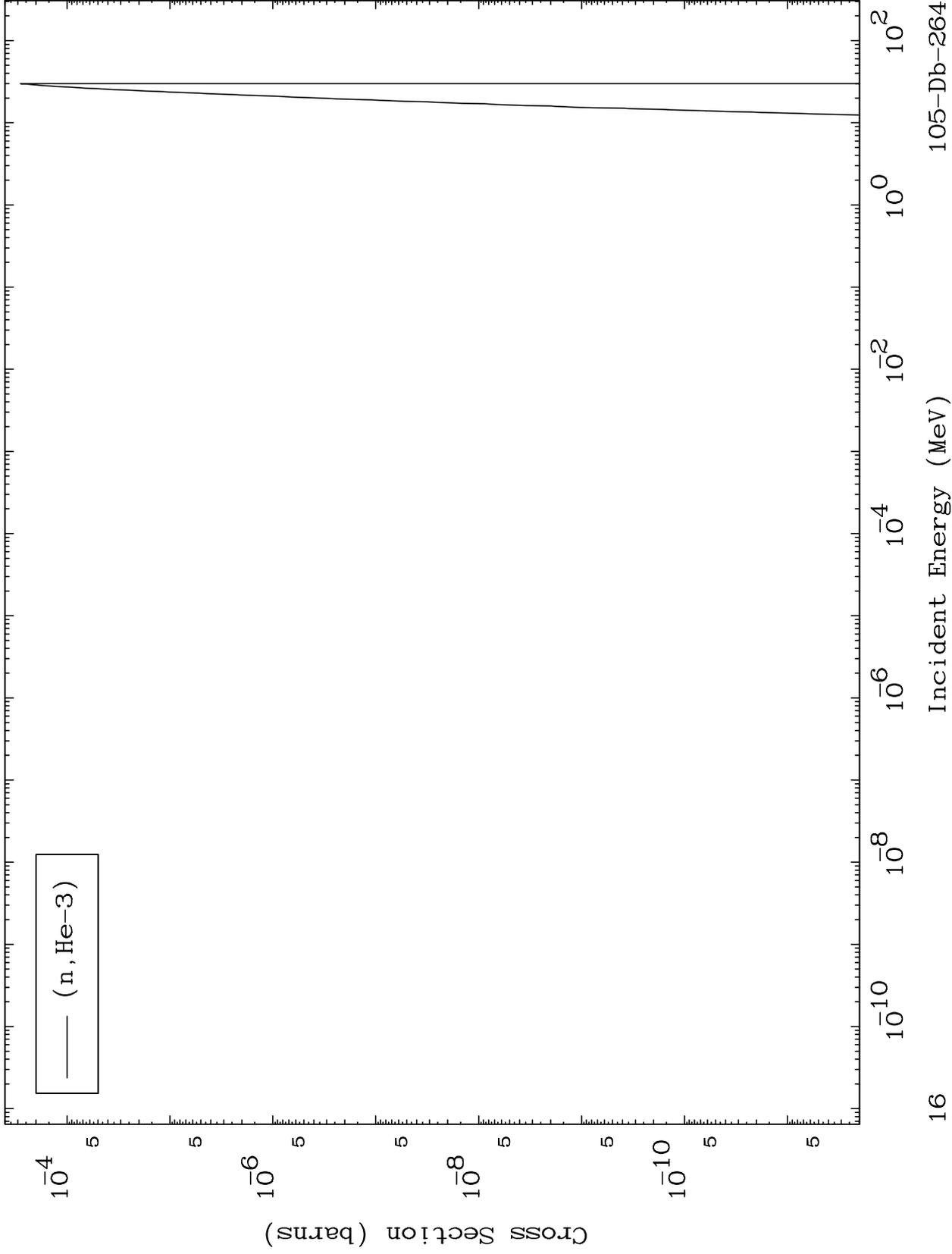
Incident Energy (MeV)

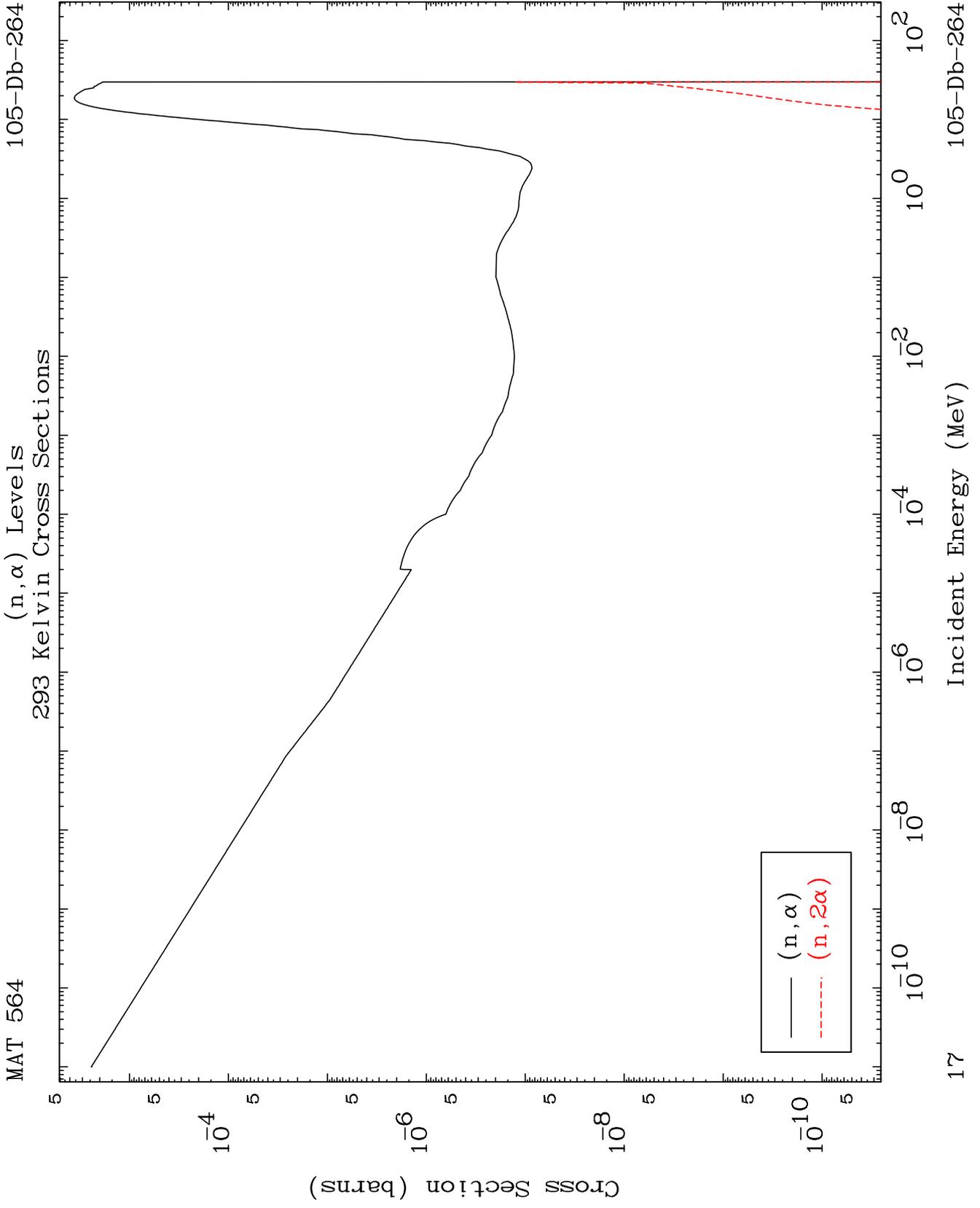
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(n,He3) Levels
293 Kelvin Cross Sections

105-Db-264

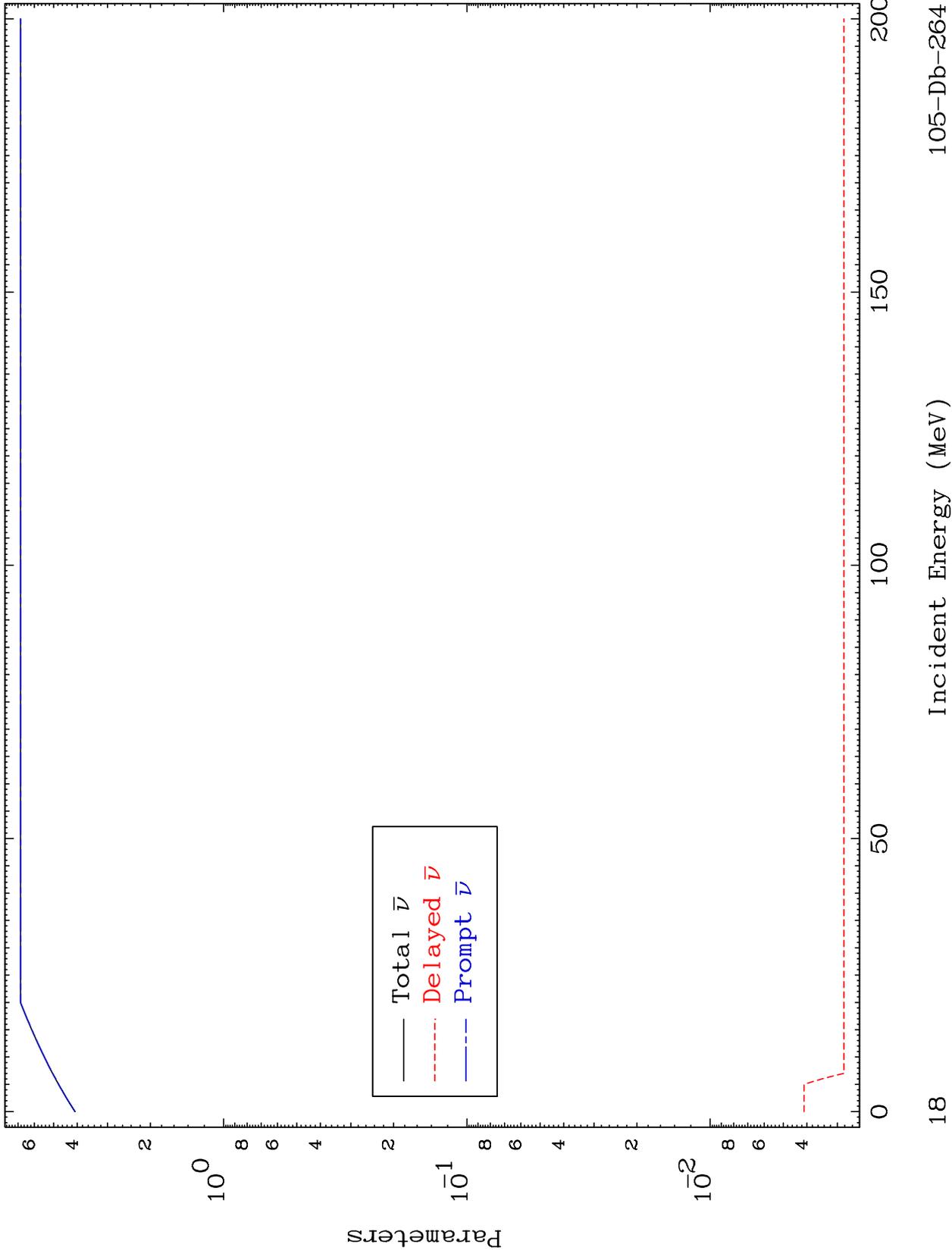




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Energy Release
Parameters

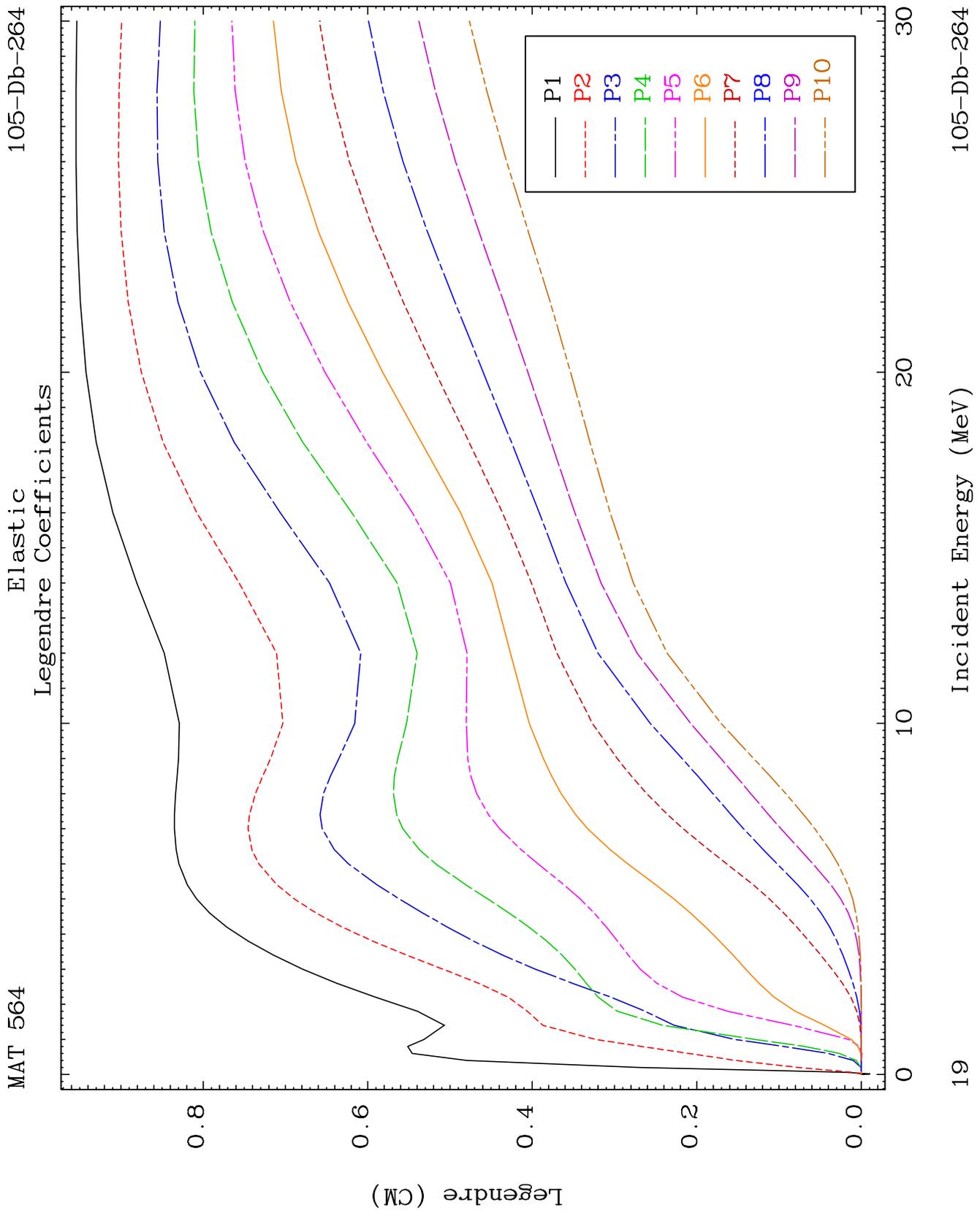
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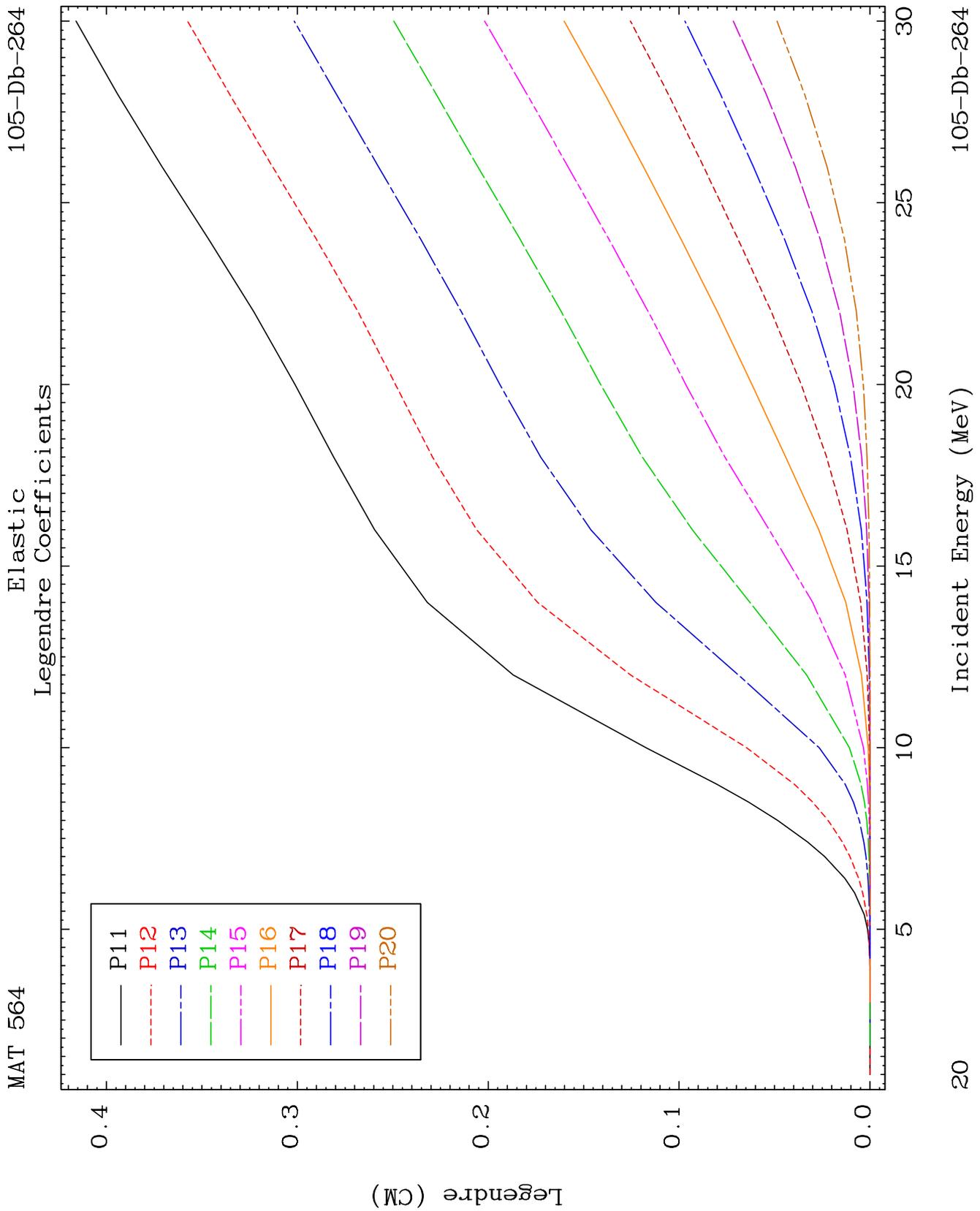


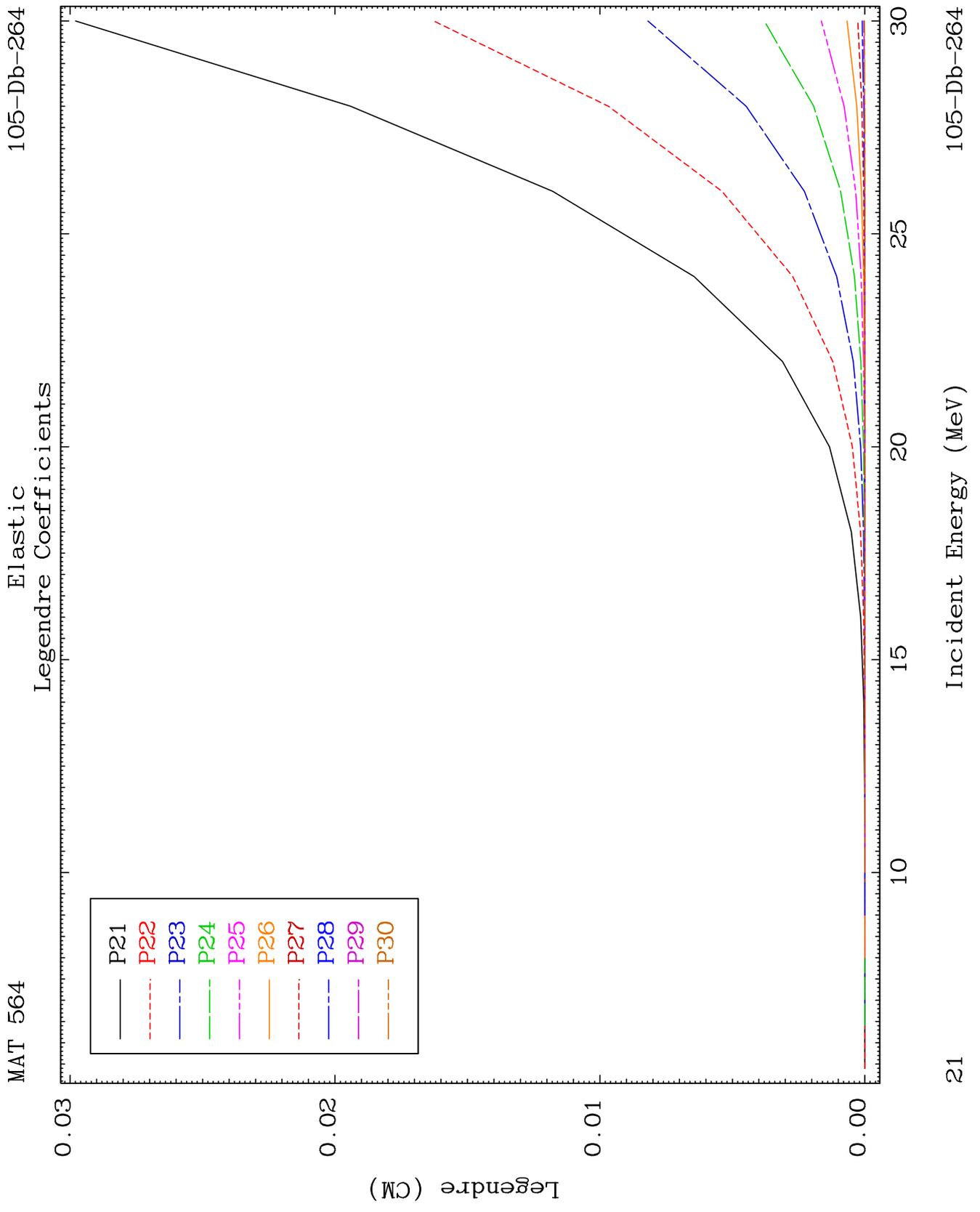
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Incident Energy (MeV)

105-Db-264



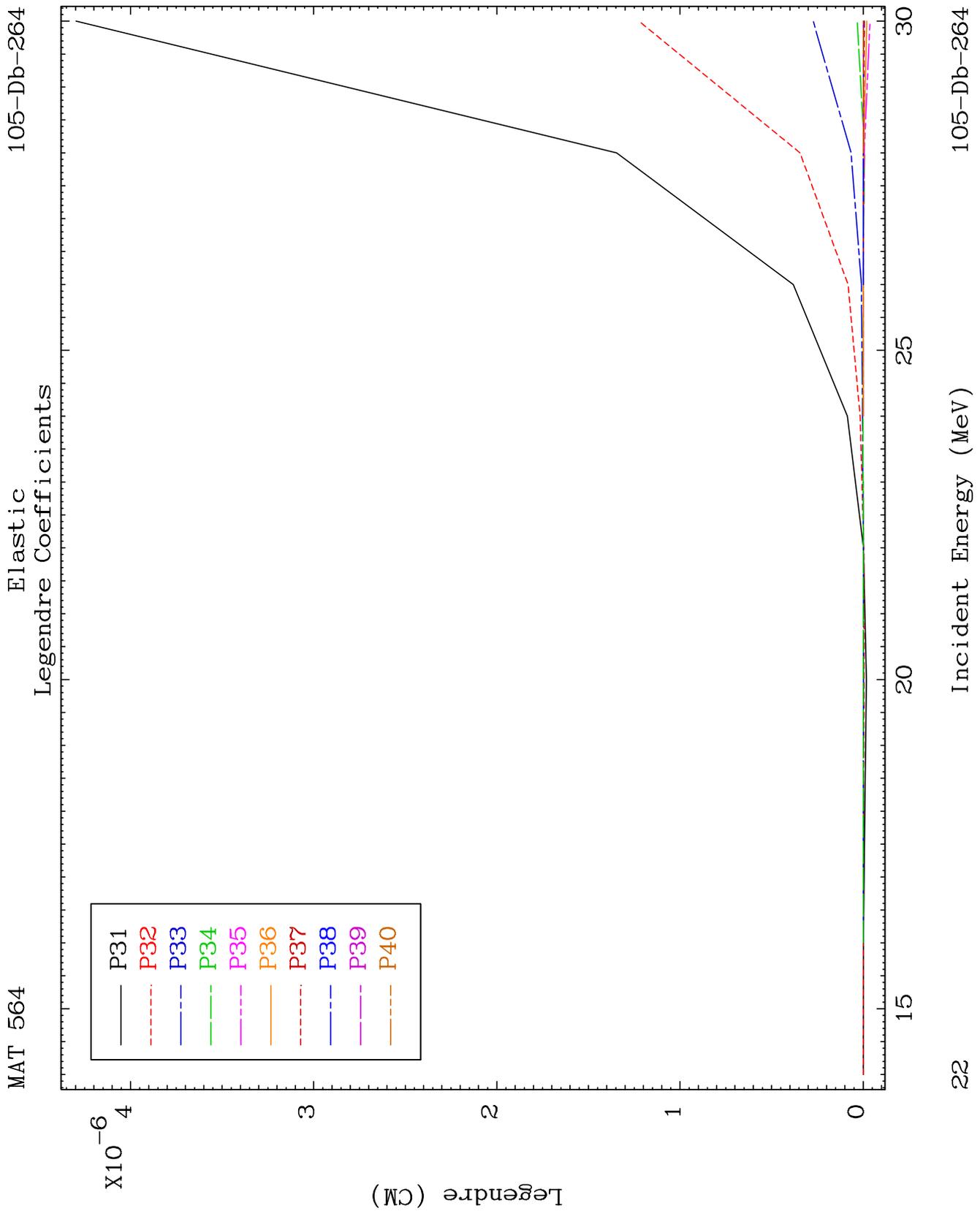




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Elastic Legendre Coefficients

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Elastic Legendre Coefficients

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22

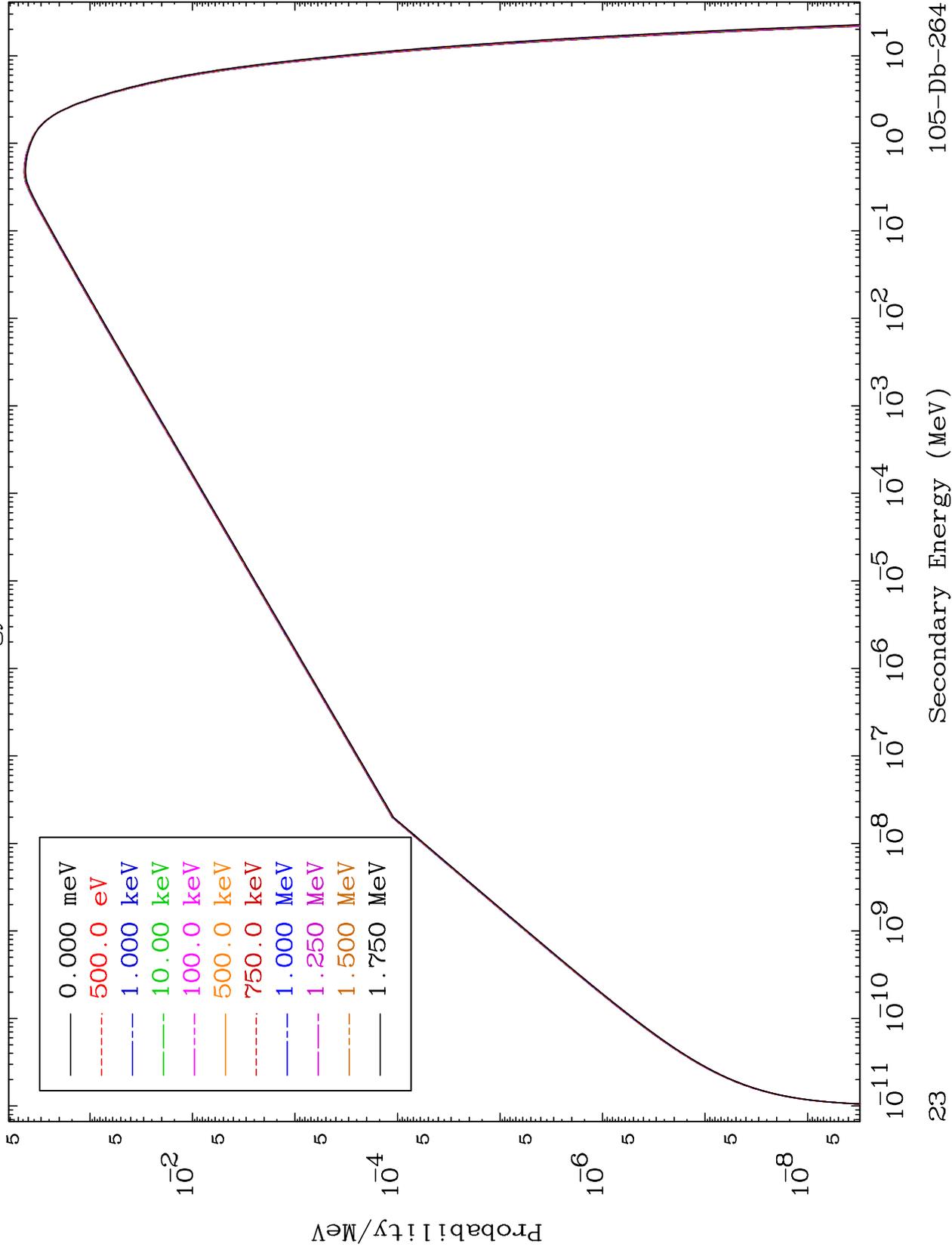
Incident Energy (MeV)

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MAT 564

Fission Energy Distributions

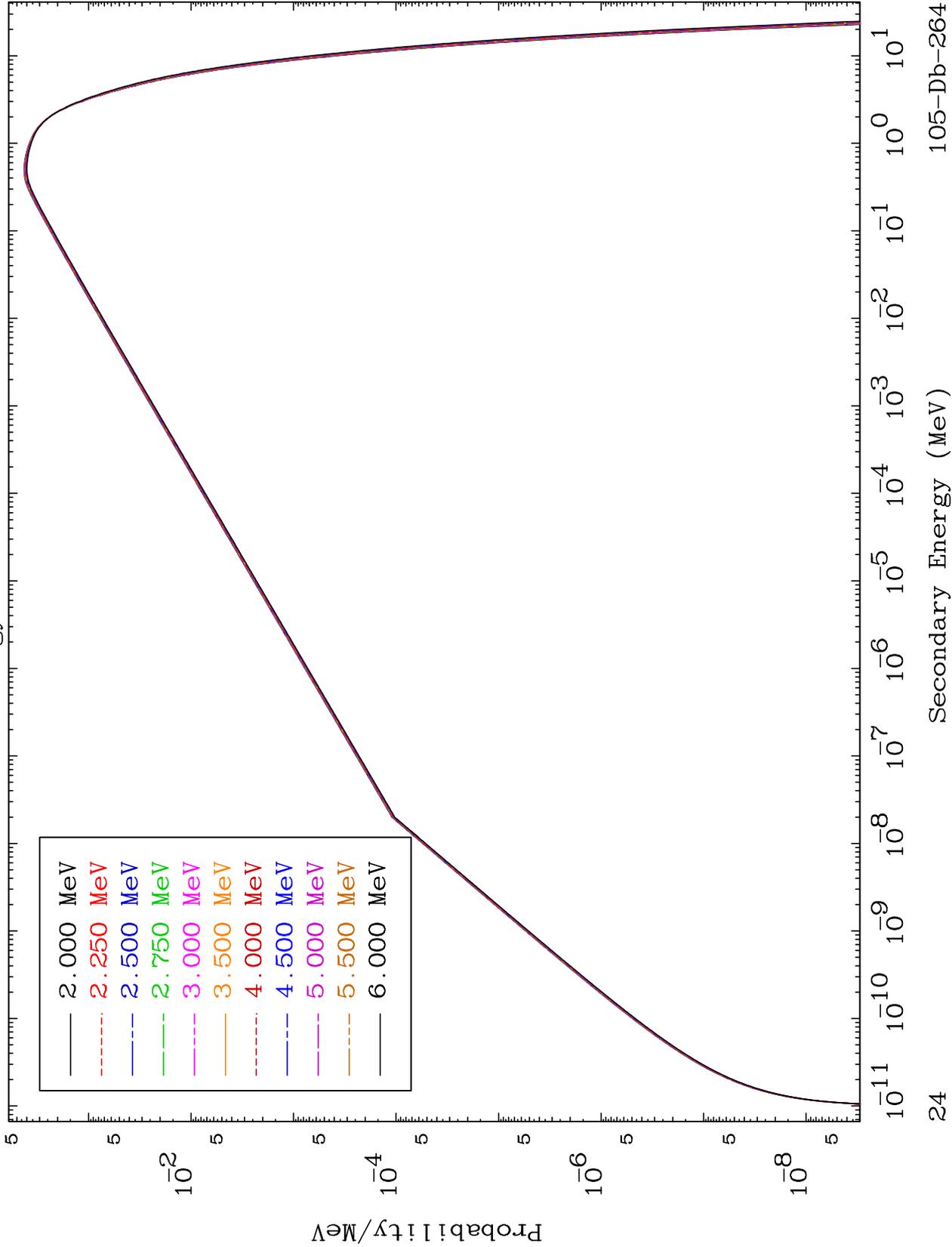
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Fission Energy Distributions

105-Db-264



105-Db-264

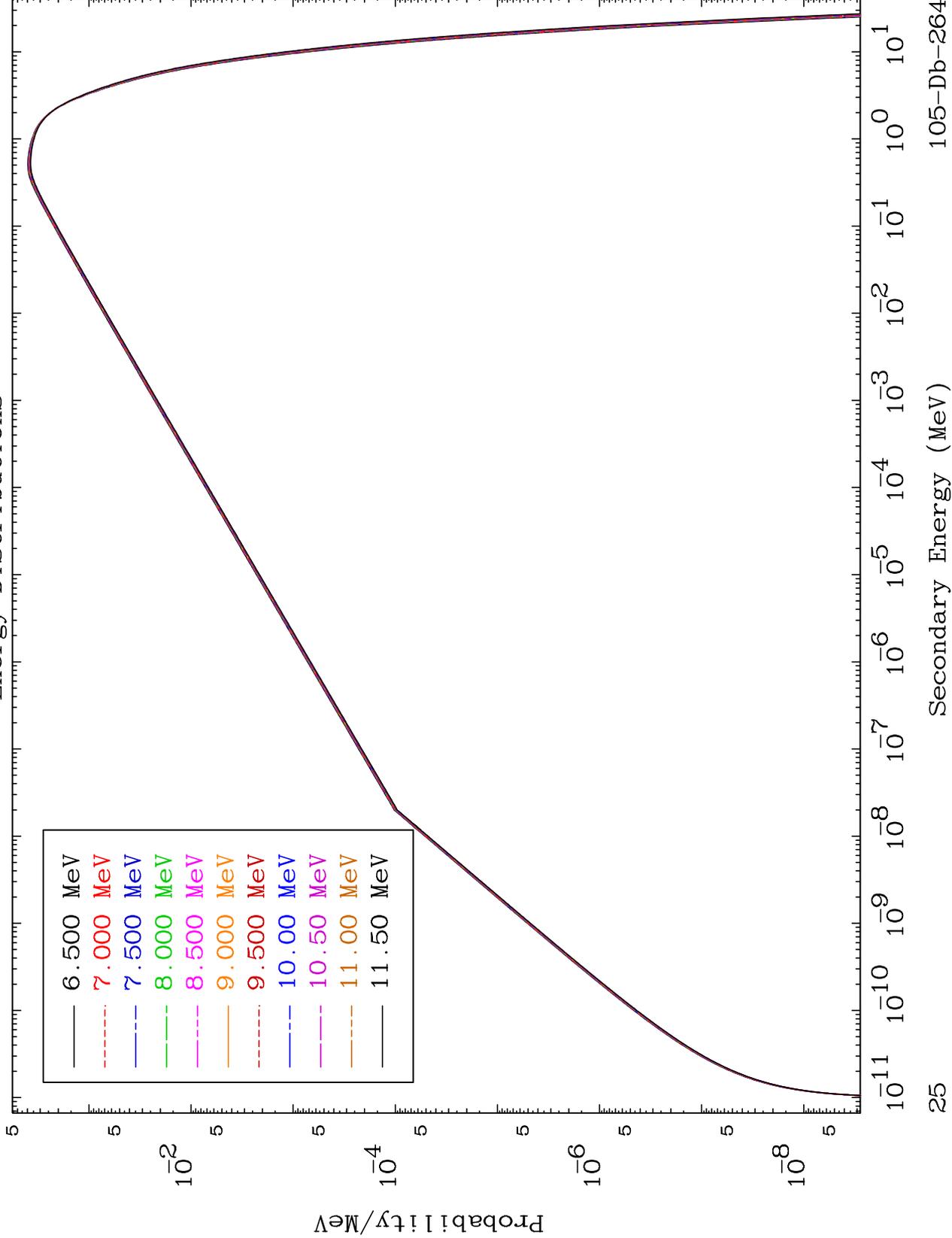
Secondary Energy (MeV)

24

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Fission Energy Distributions

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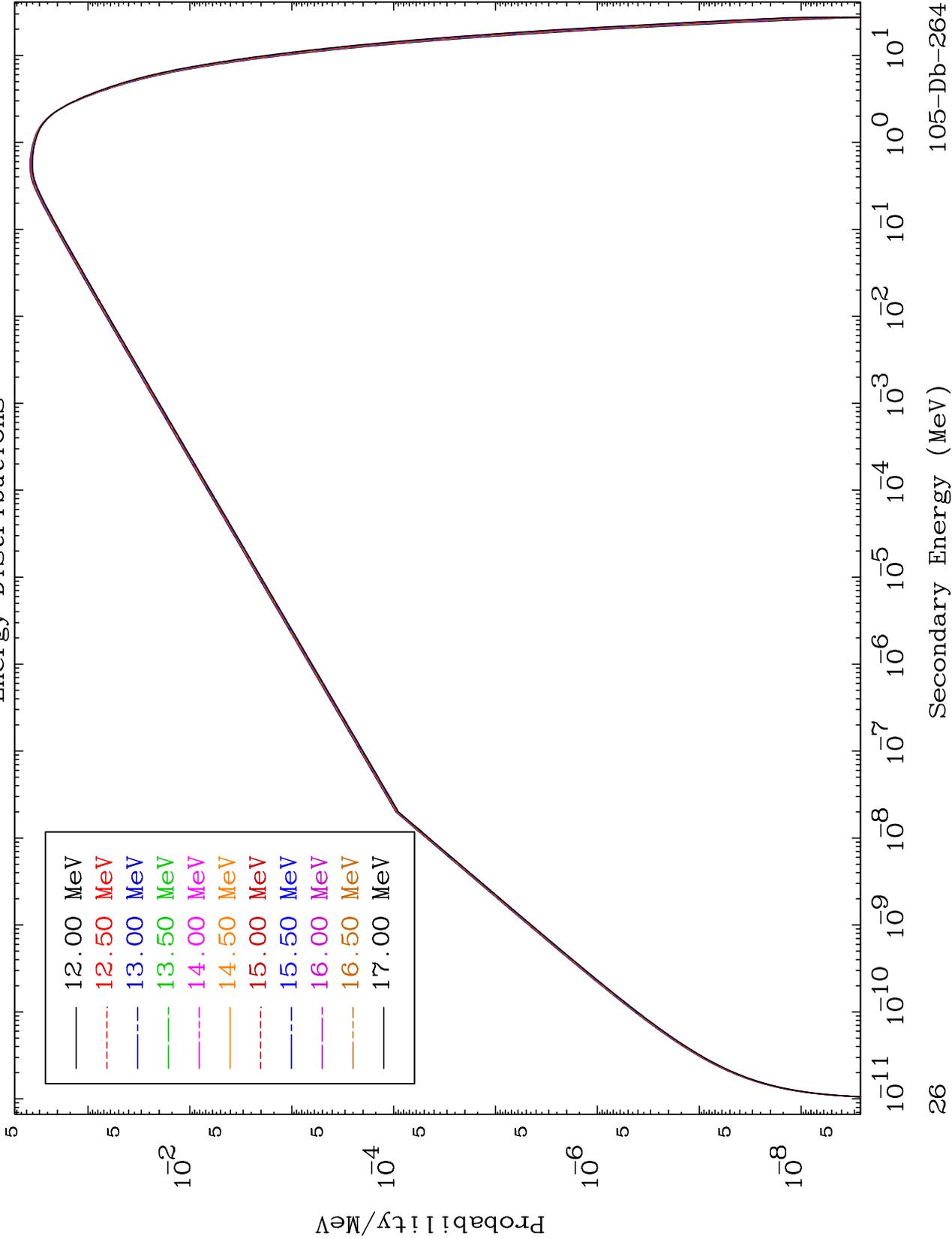


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Fission Energy Distributions

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105-Db-264

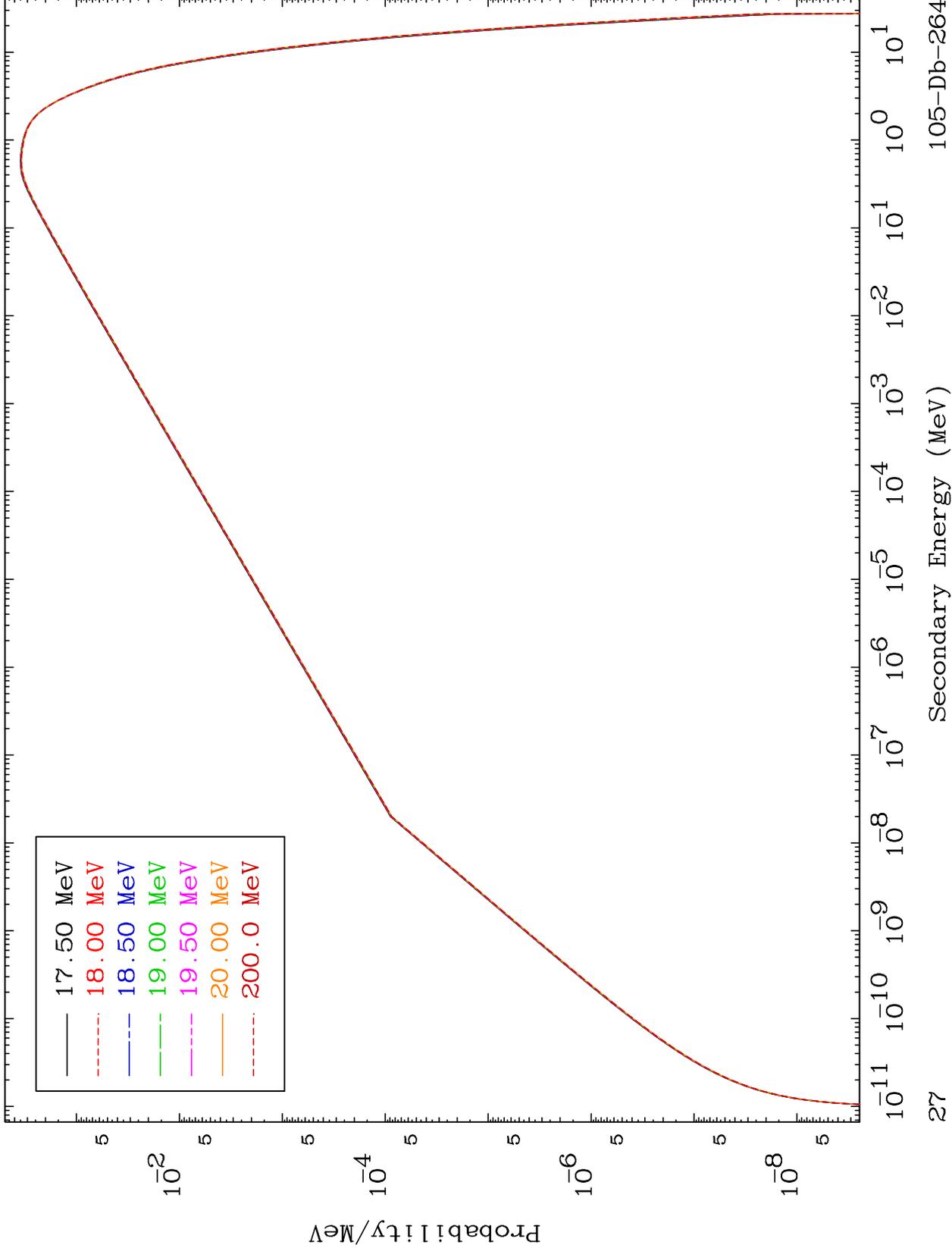
Secondary Energy (MeV)

26

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Fission Energy Distributions

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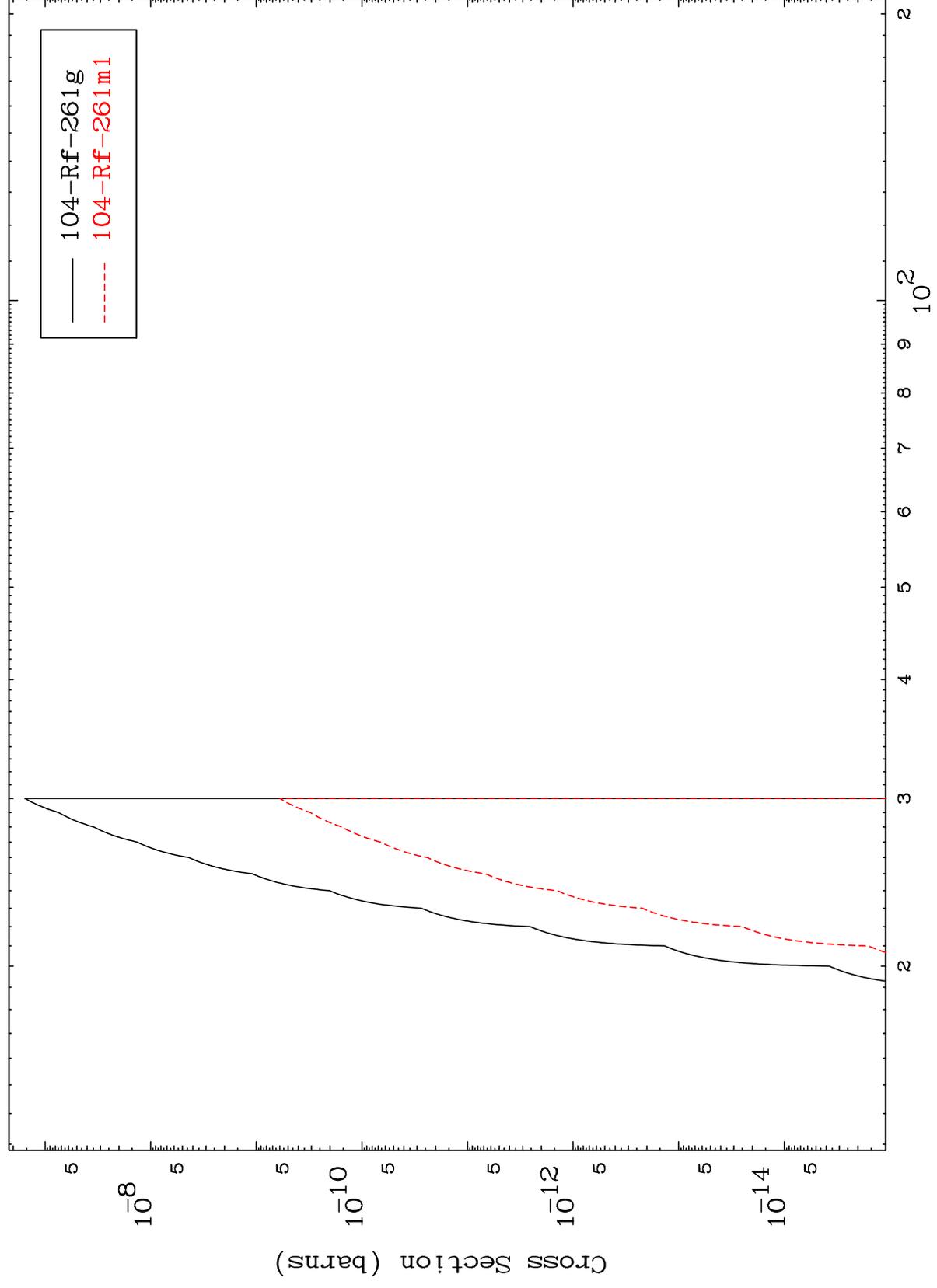


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(n,2n) d

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Radionuclide Production Cross Section



28

Incident Energy (MeV)

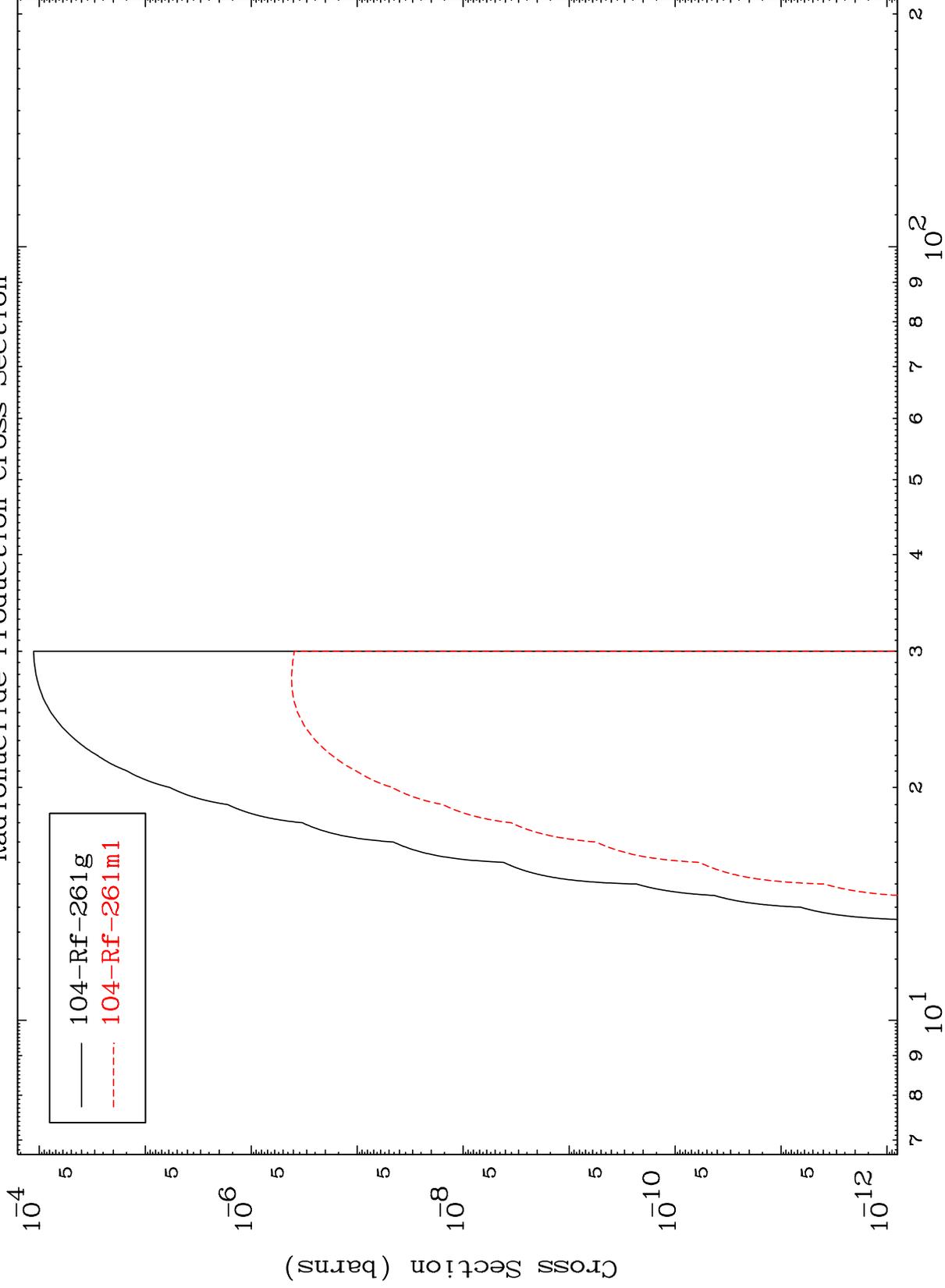
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(n,n') t

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Radionuclide Production Cross Section



29

Incident Energy (MeV)

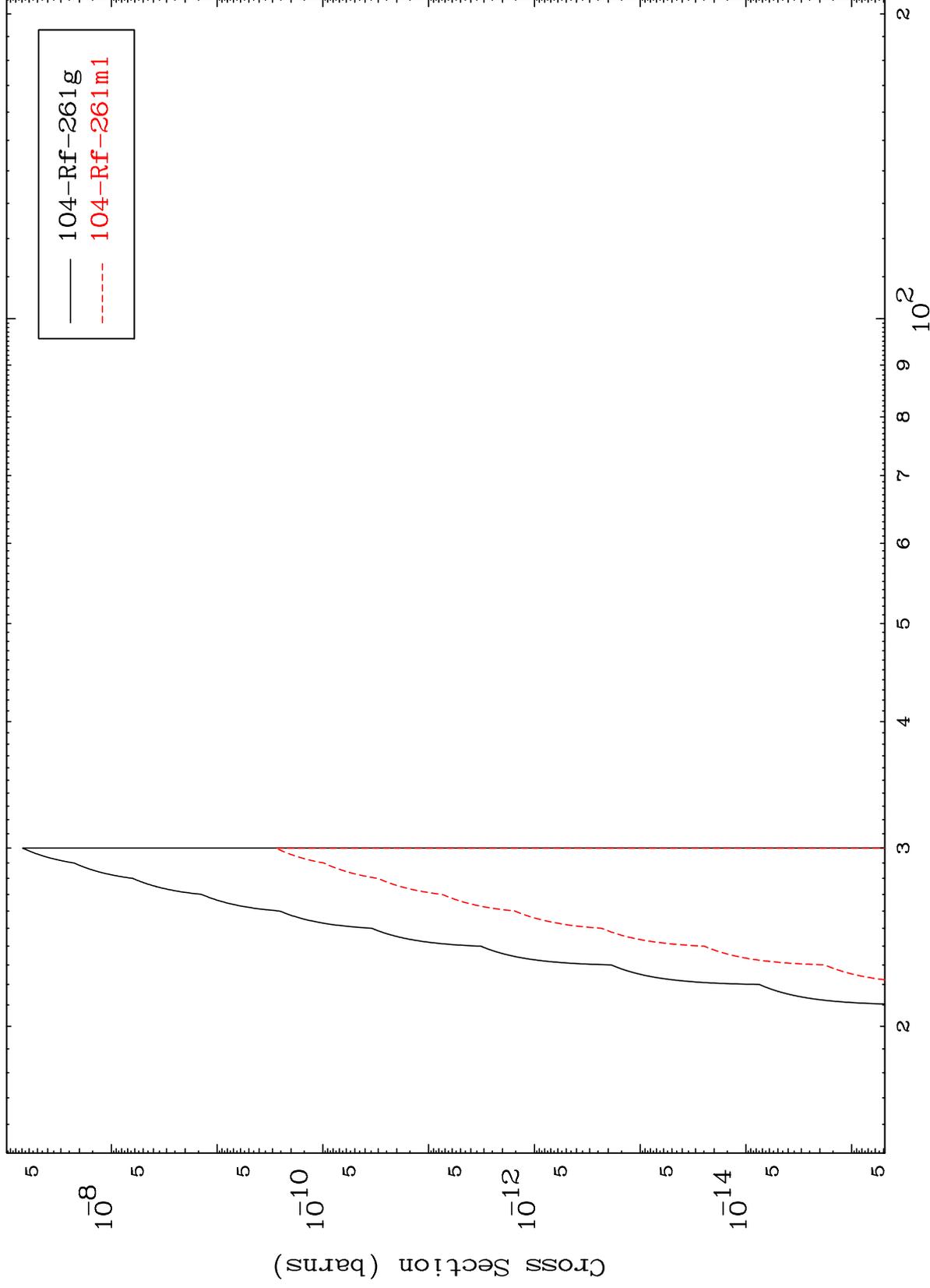
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(n,3n) p

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Radionuclide Production Cross Section



30

Incident Energy (MeV)

105-Db-264