

Program EVALPLOT  
(Version 2021-1)

by

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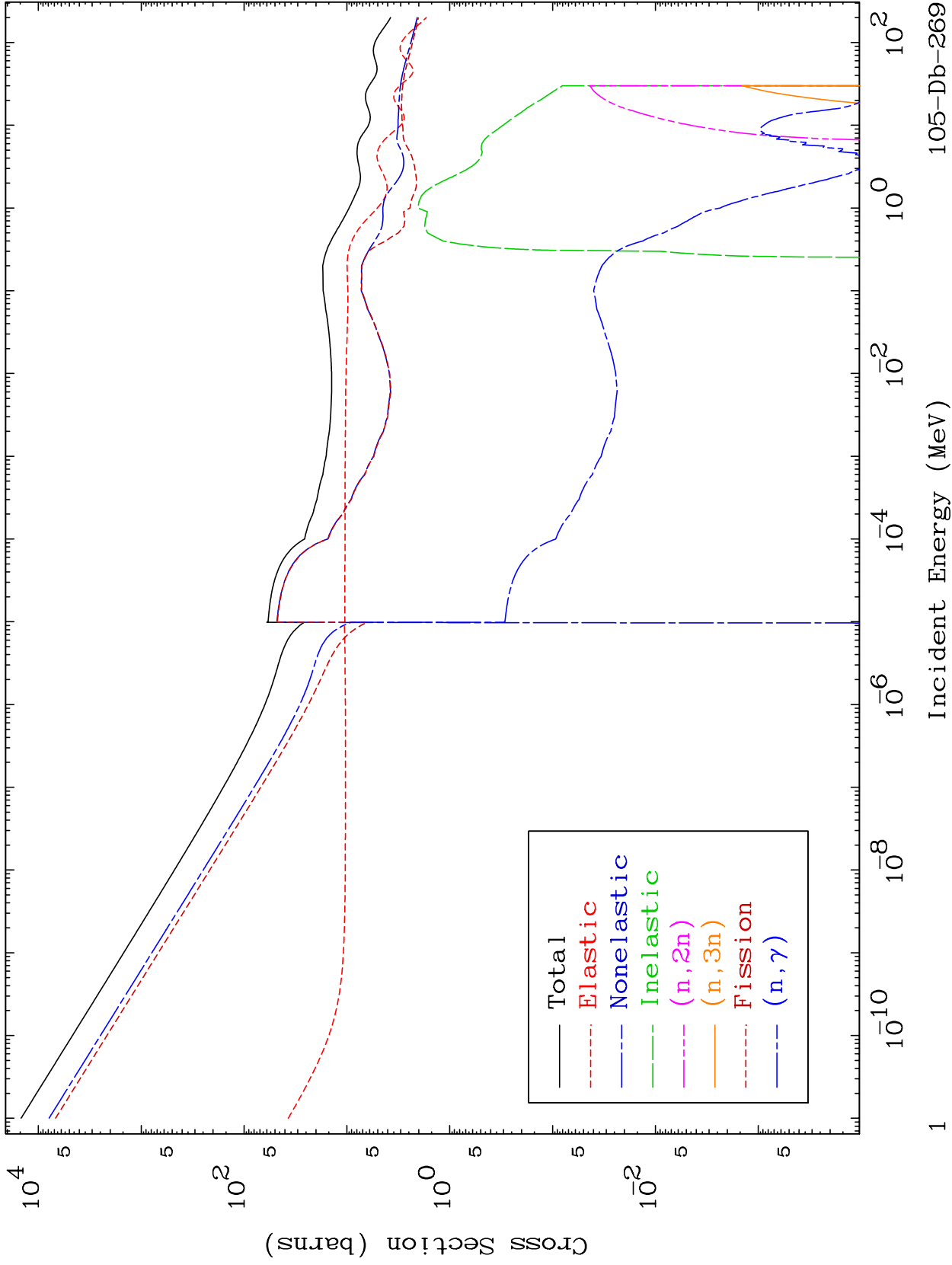
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Press Mouse Button to Start

MAT 569

Neutron Major  
293 Kelvin Cross Sections

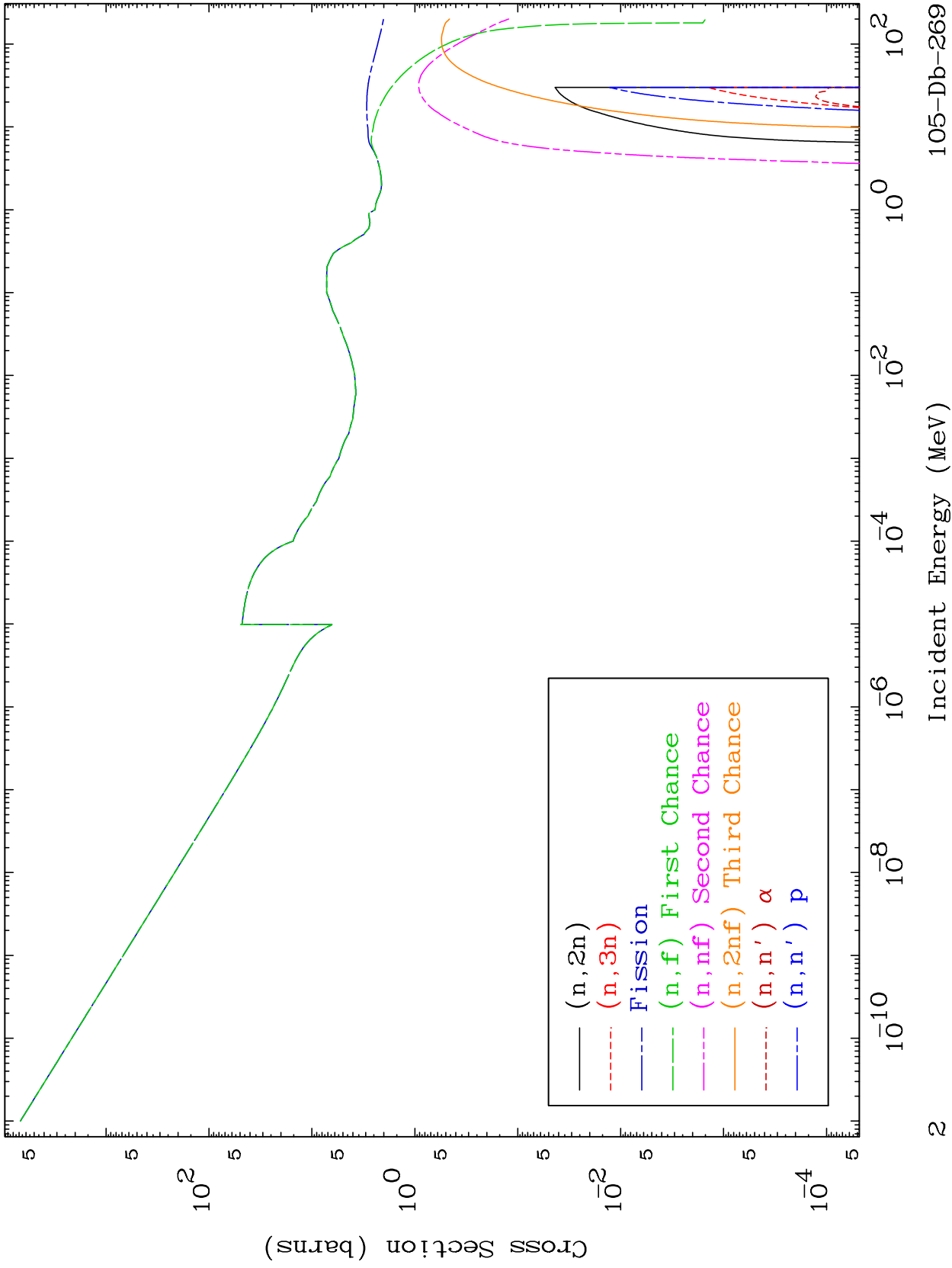
105-Db-269



MAT 569

Neutron Absorption  
293 Kelvin Cross Sections

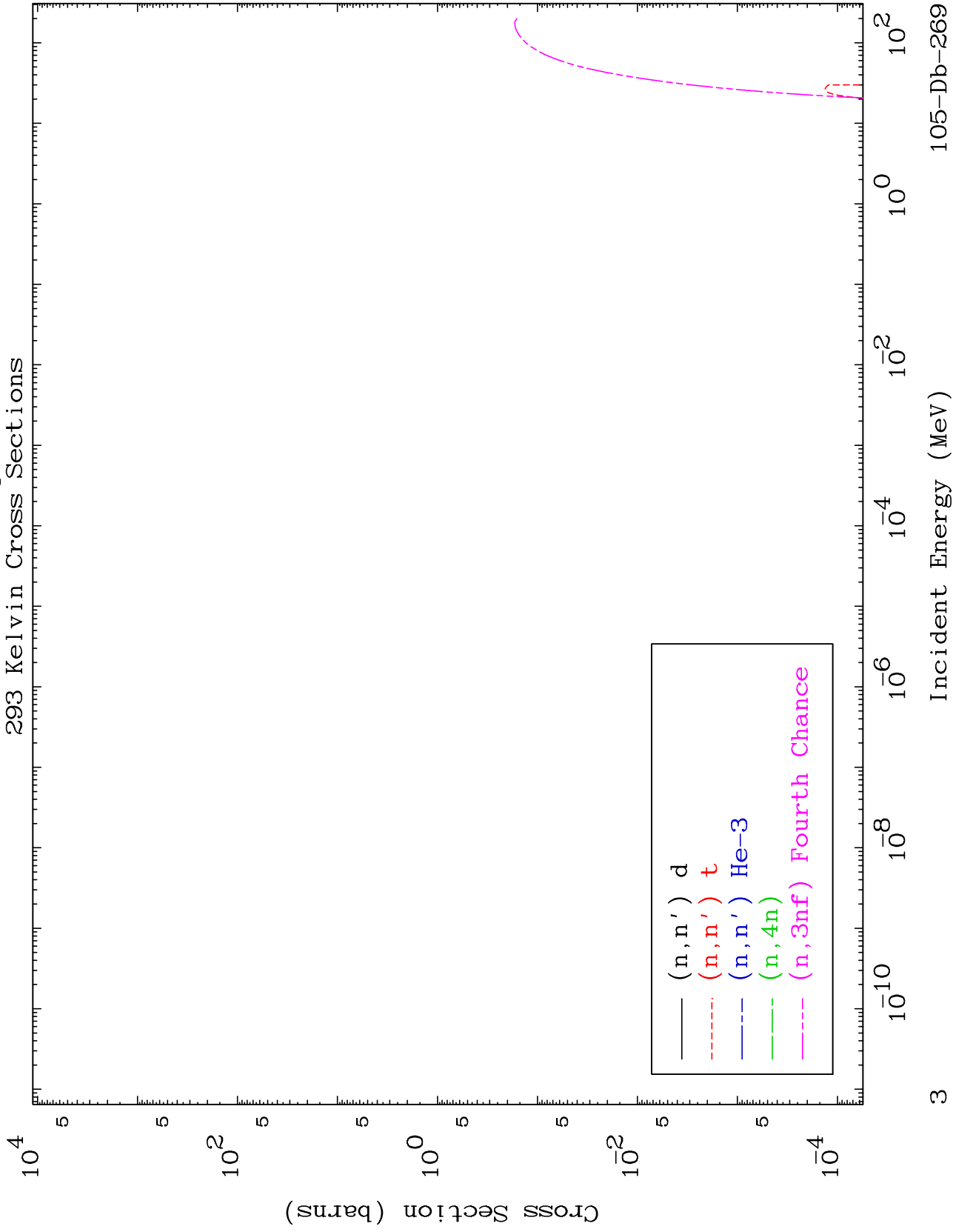
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MAT 569

Neutron Absorption  
293 Kelvin Cross Sections

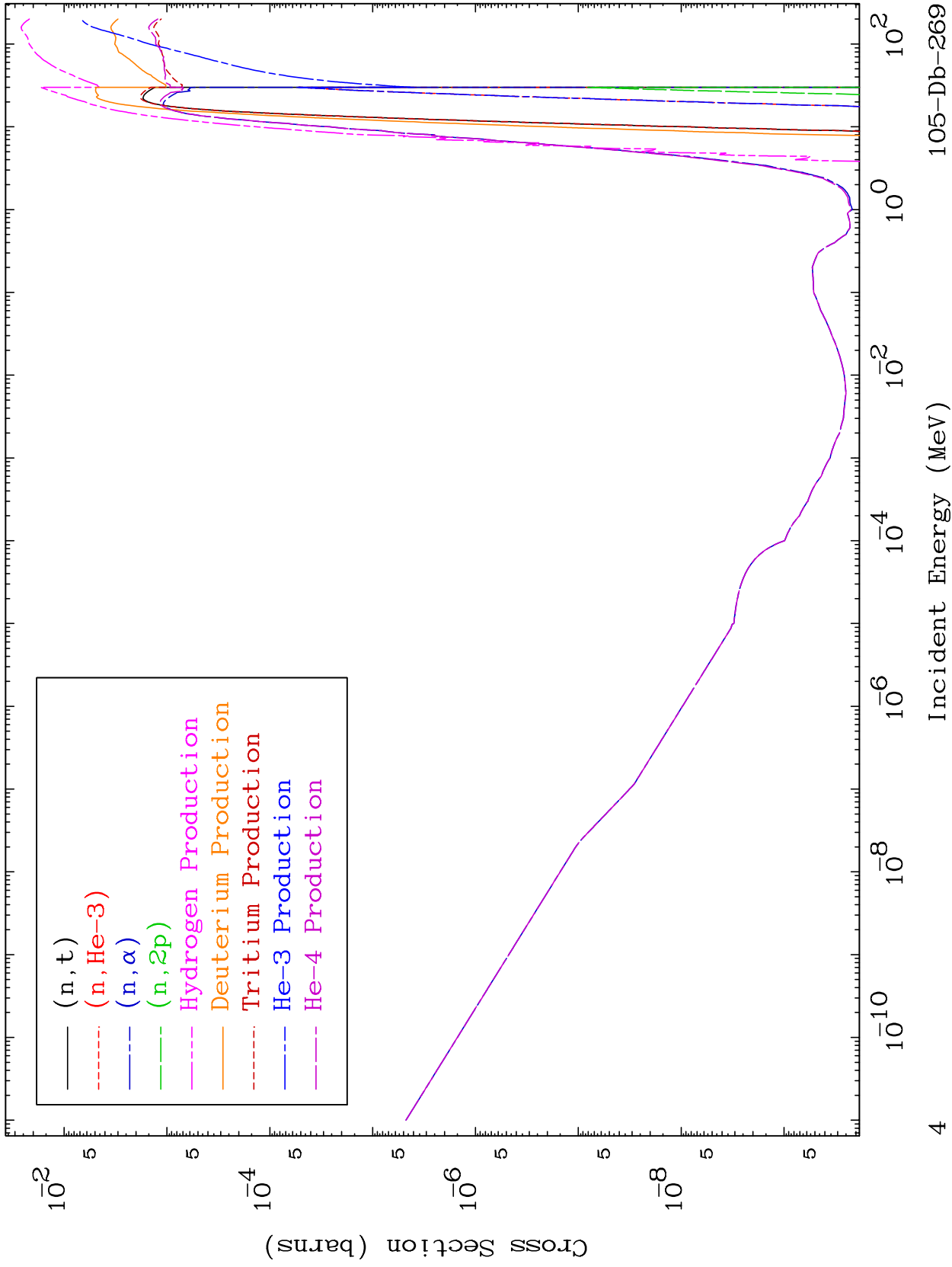
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MAT 569

Neutron Absorption  
293 Kelvin Cross Sections

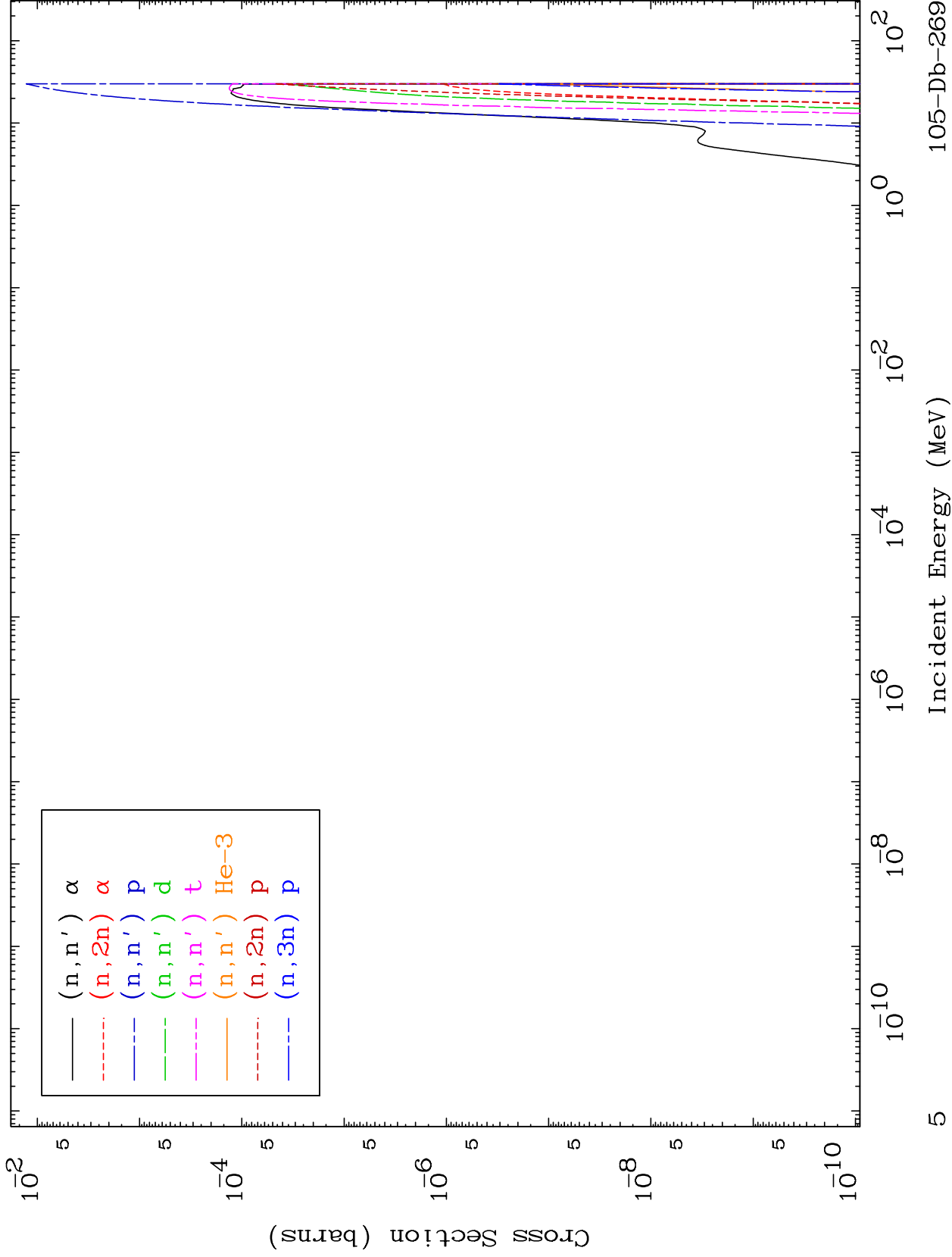
105-Db-269



MAT 569

Charged Particle  
293 Kelvin Cross Sections

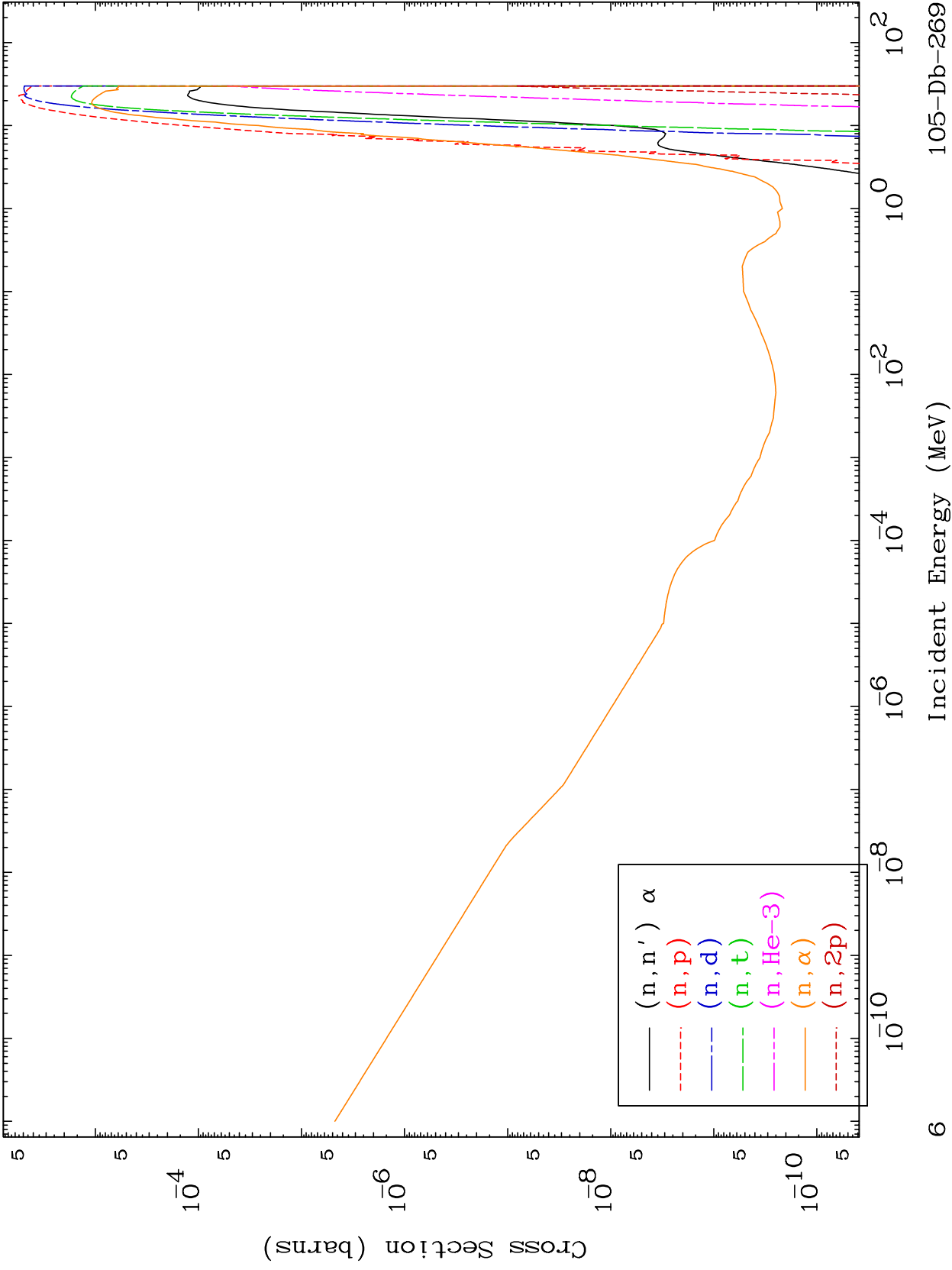
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Charged Particle  
293 Kelvin Cross Sections

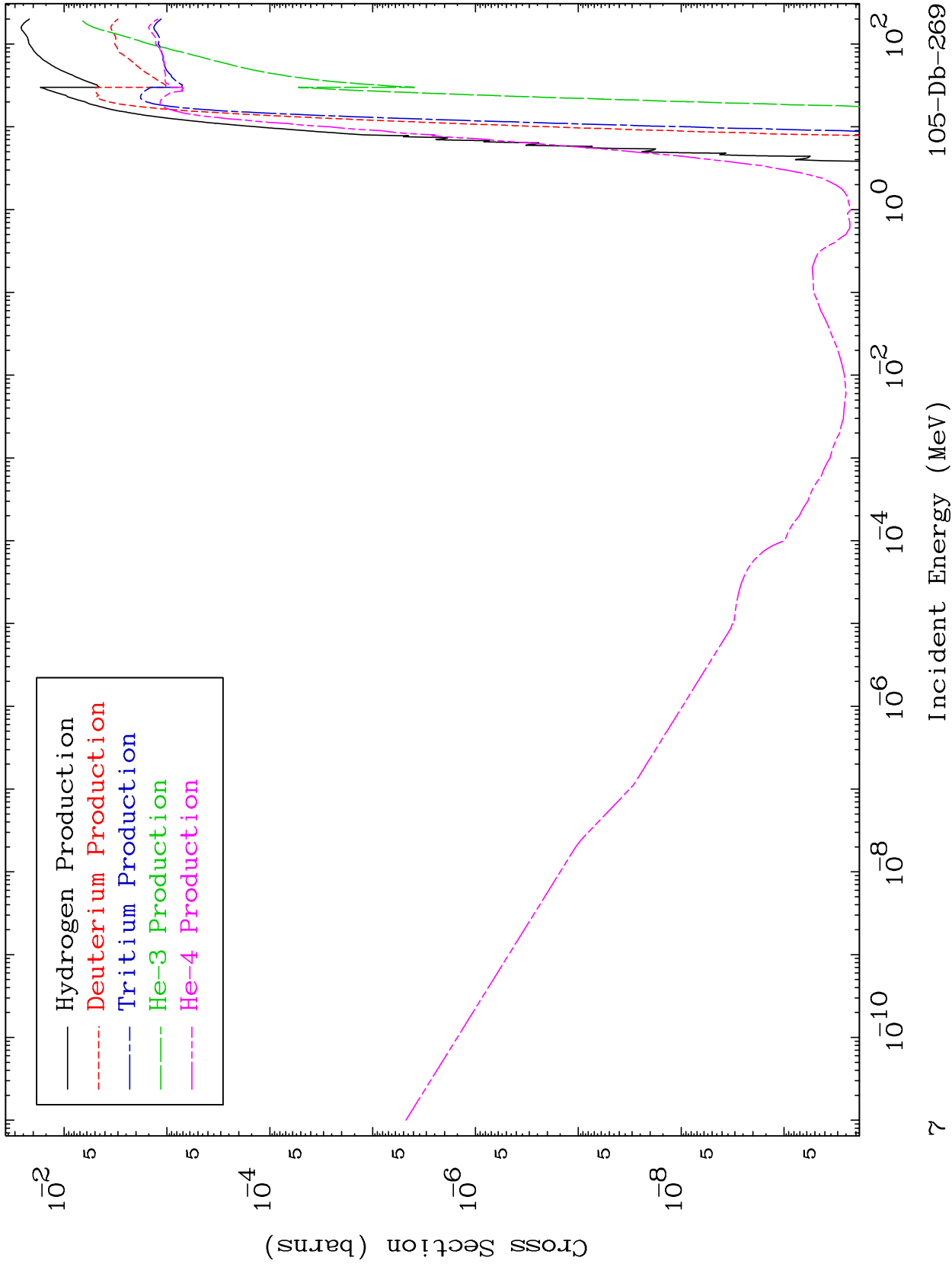
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Particle Production  
293 Kelvin Cross Sections

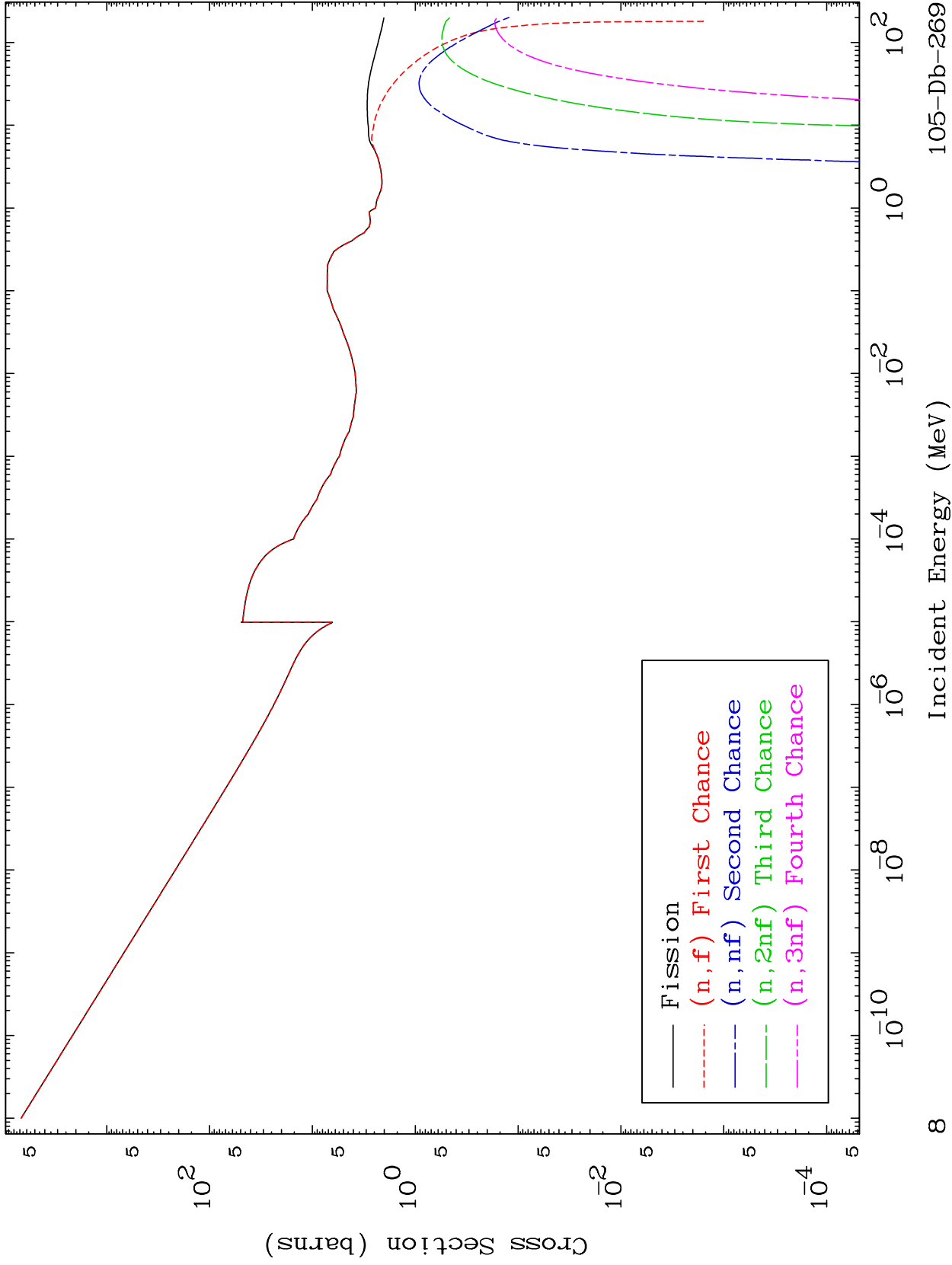
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Fission  
293 Kelvin Cross Sections

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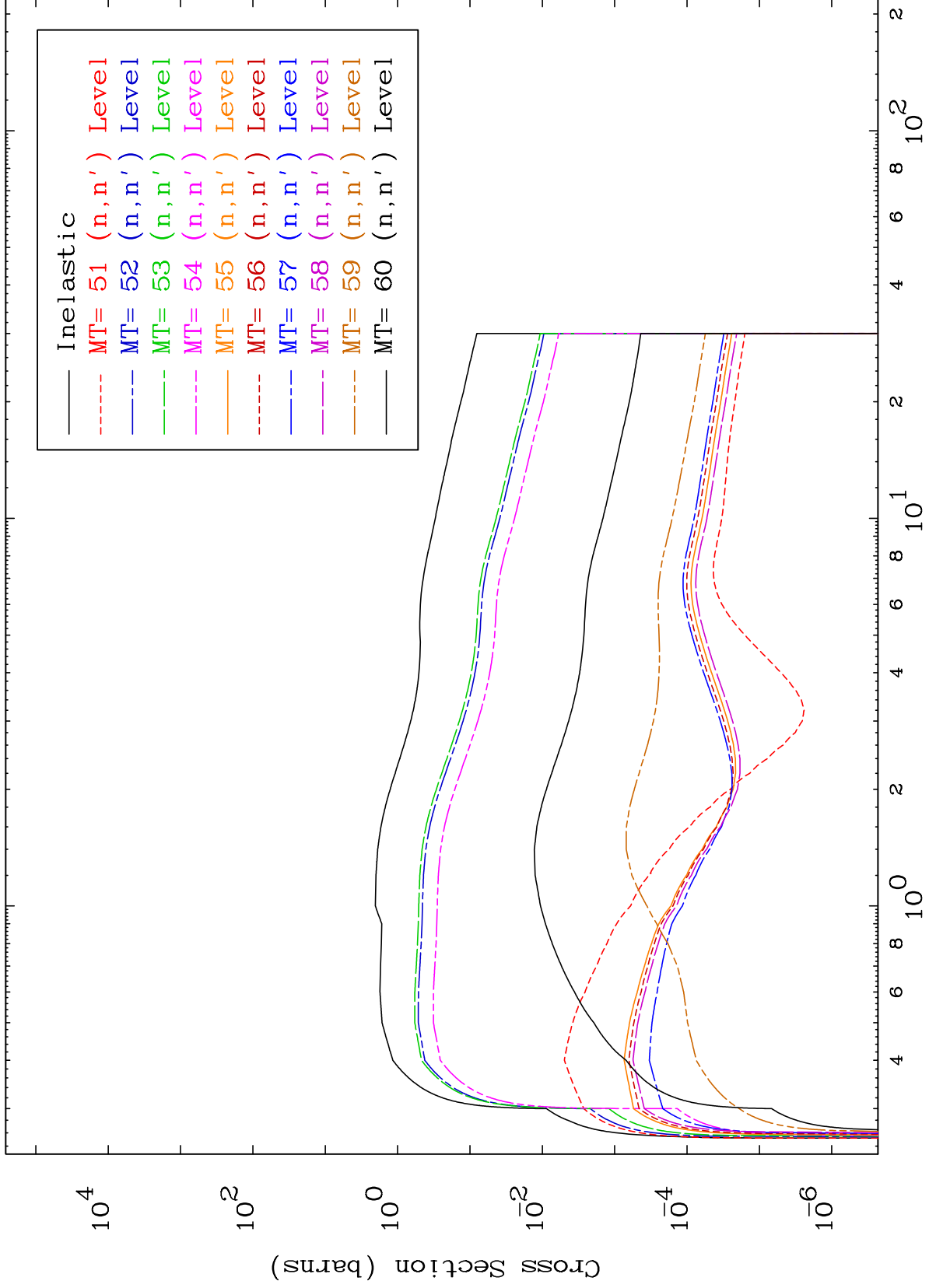


MAT 569

(n,n') Levels

105-Db-269

293 Kelvin Cross Sections

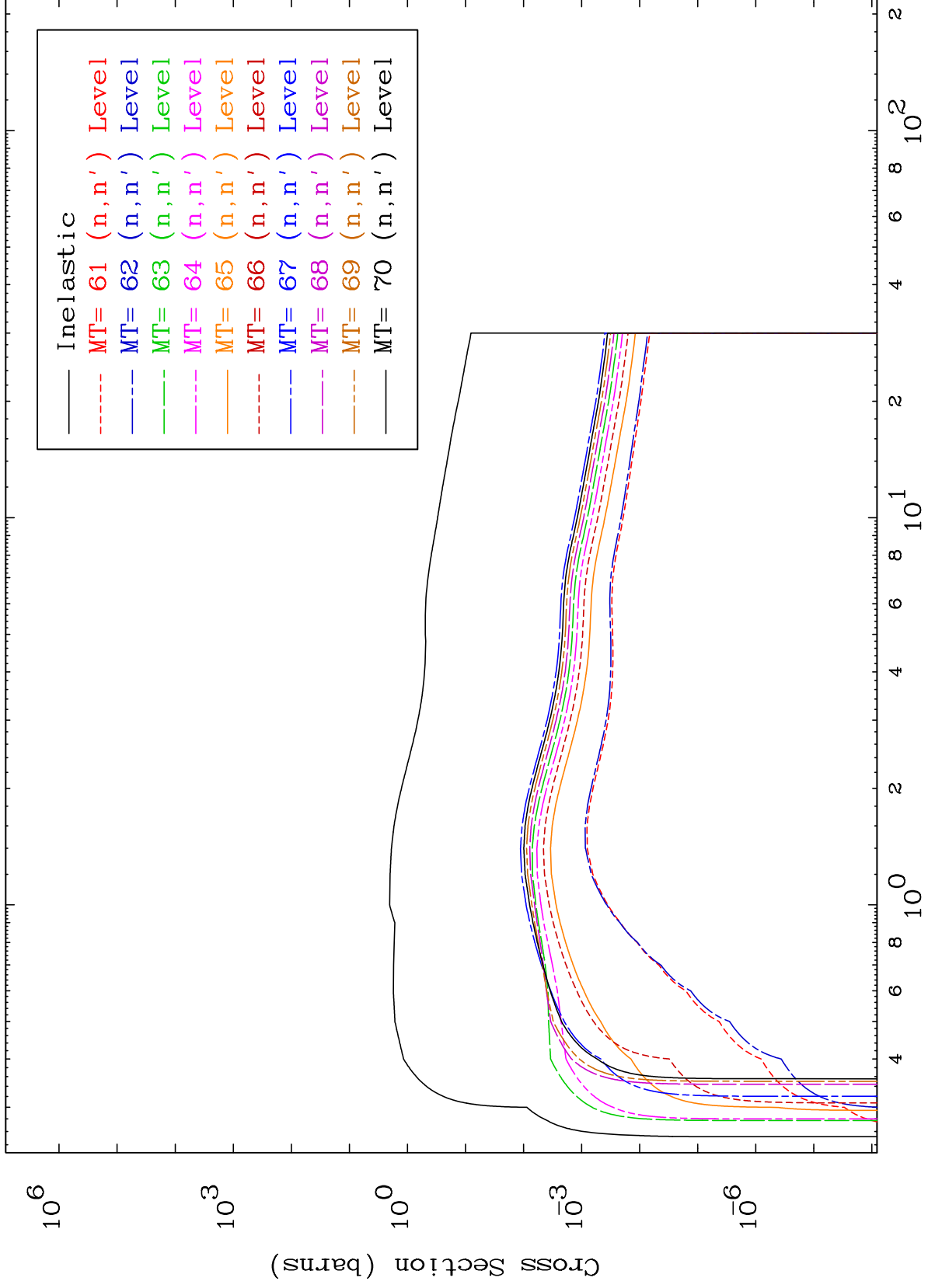


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(n,n') Levels

105-Db-269

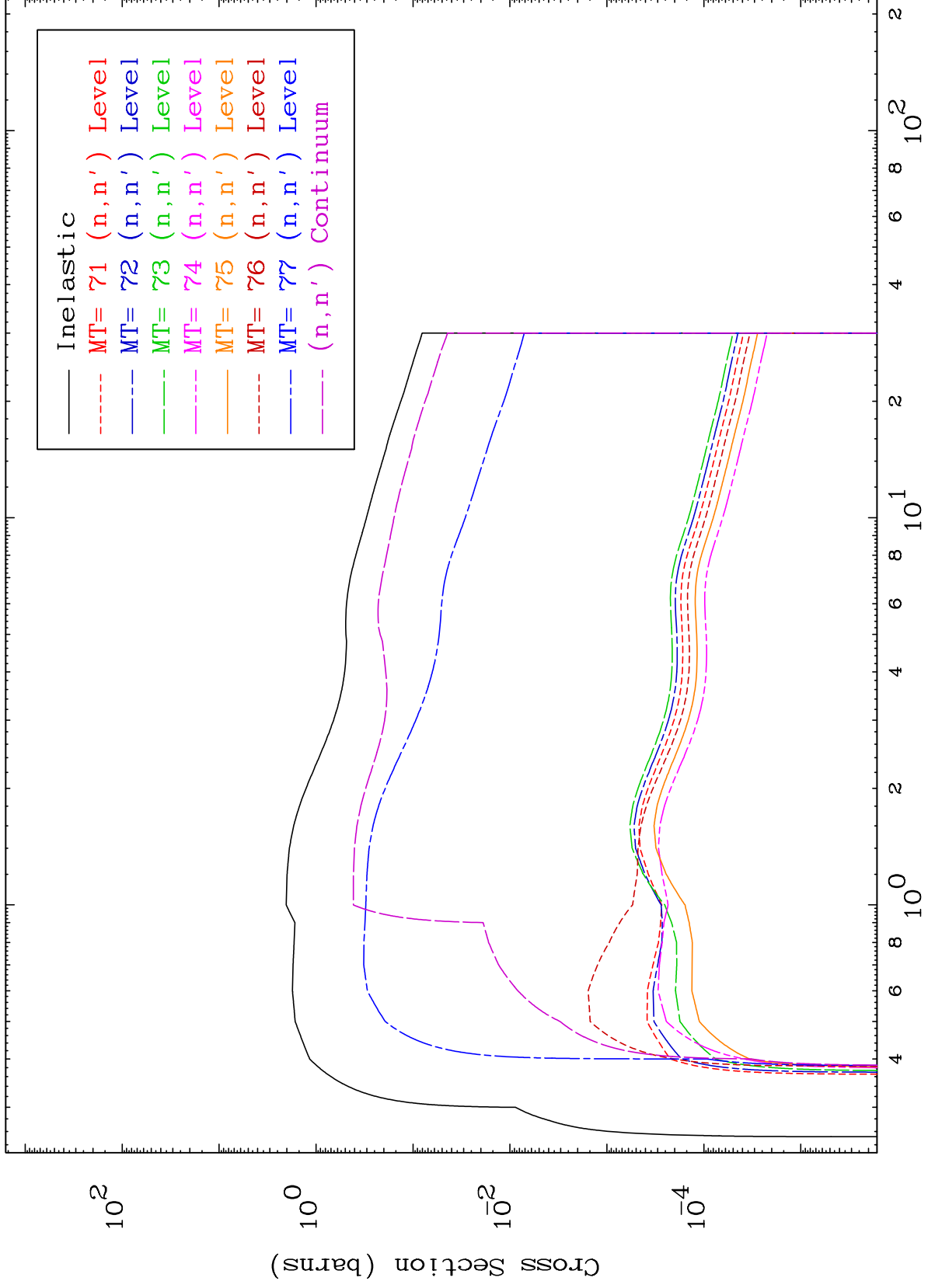
293 Kelvin Cross Sections



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Incident Energy (MeV)

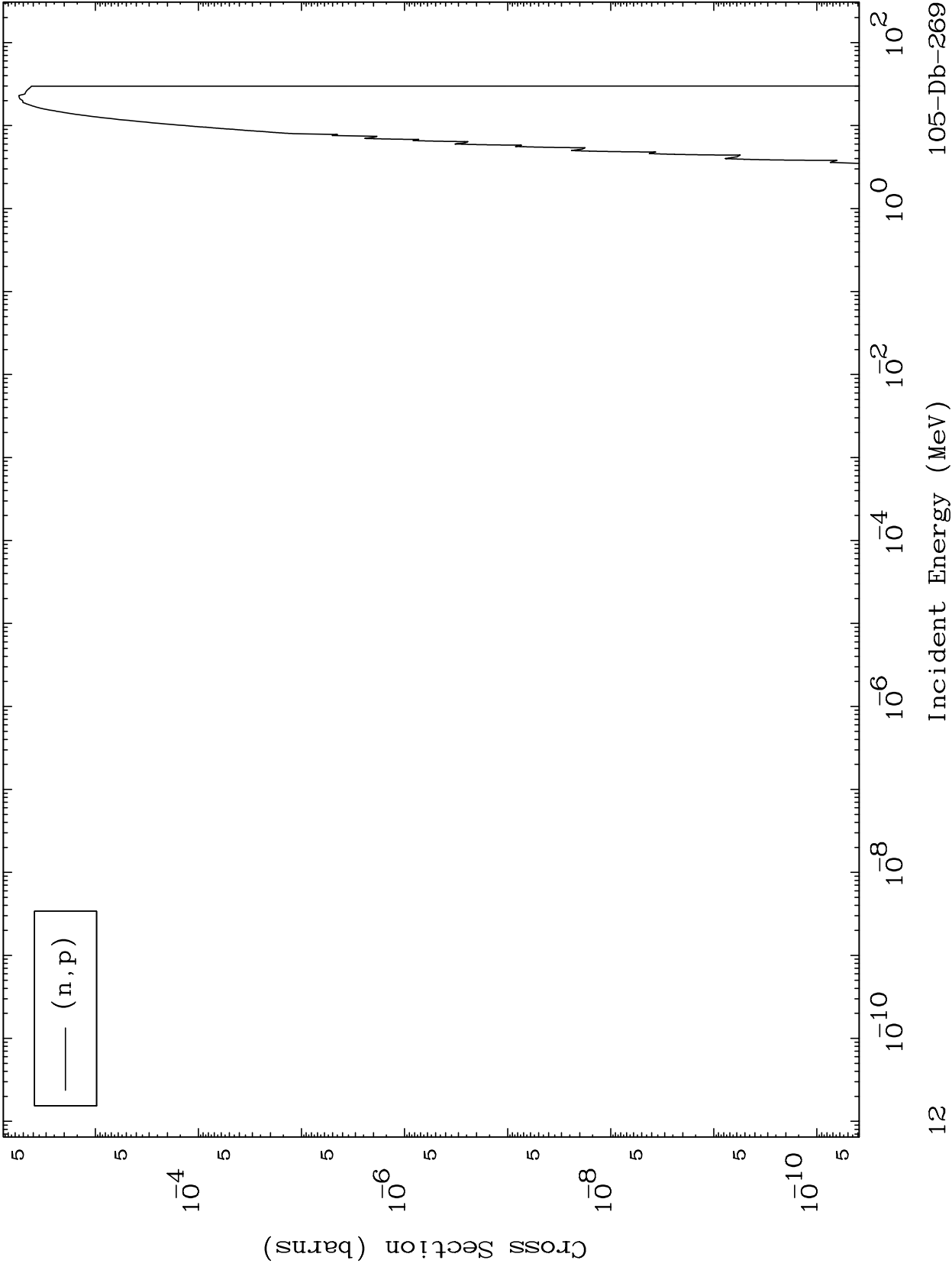
105-Db-269



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(n,p) Levels  
293 Kelvin Cross Sections

105-Db-269



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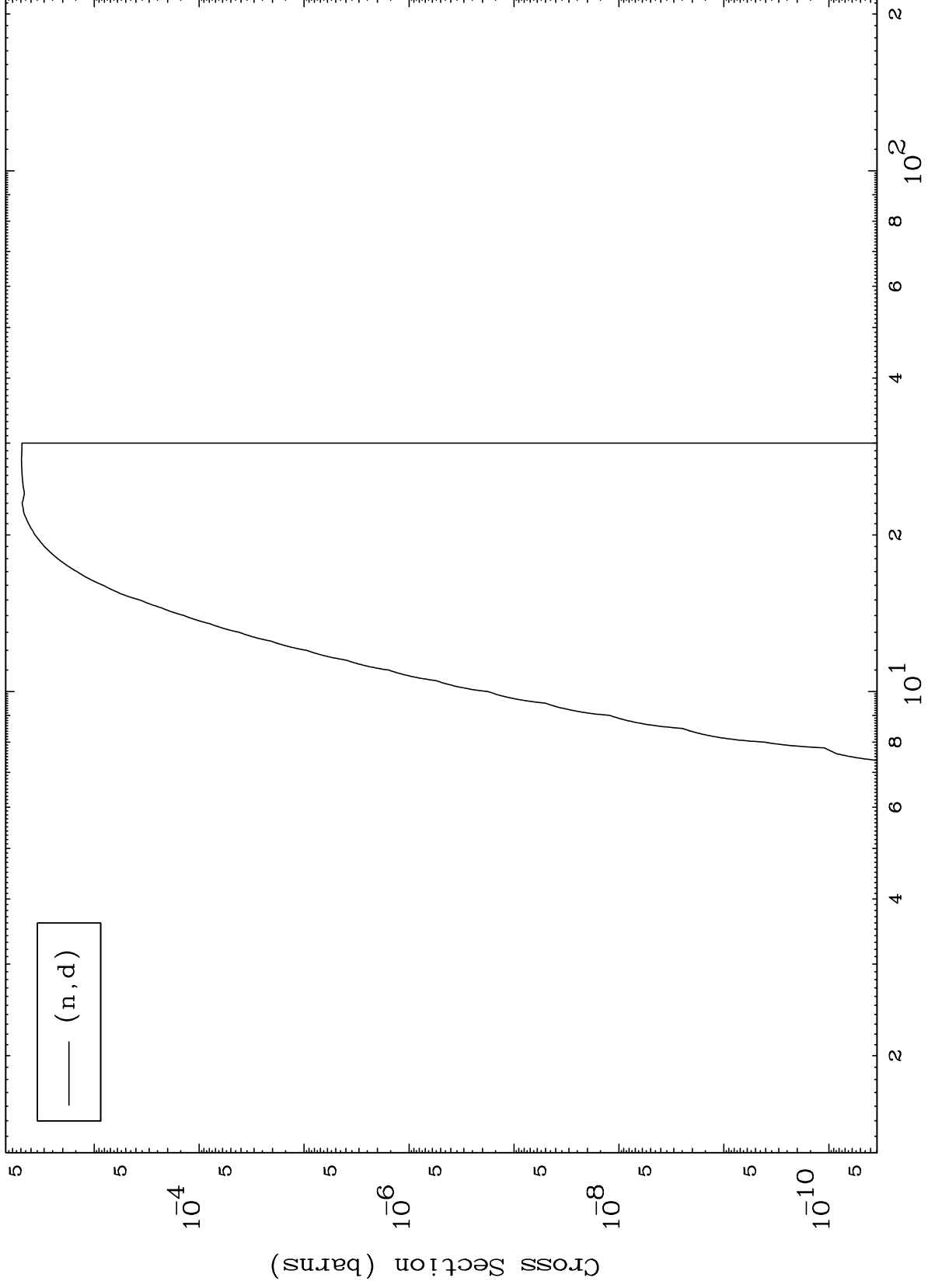
Incident Energy (MeV)

105-Db-269

MAT 569

(n,d) Levels  
293 Kelvin Cross Sections

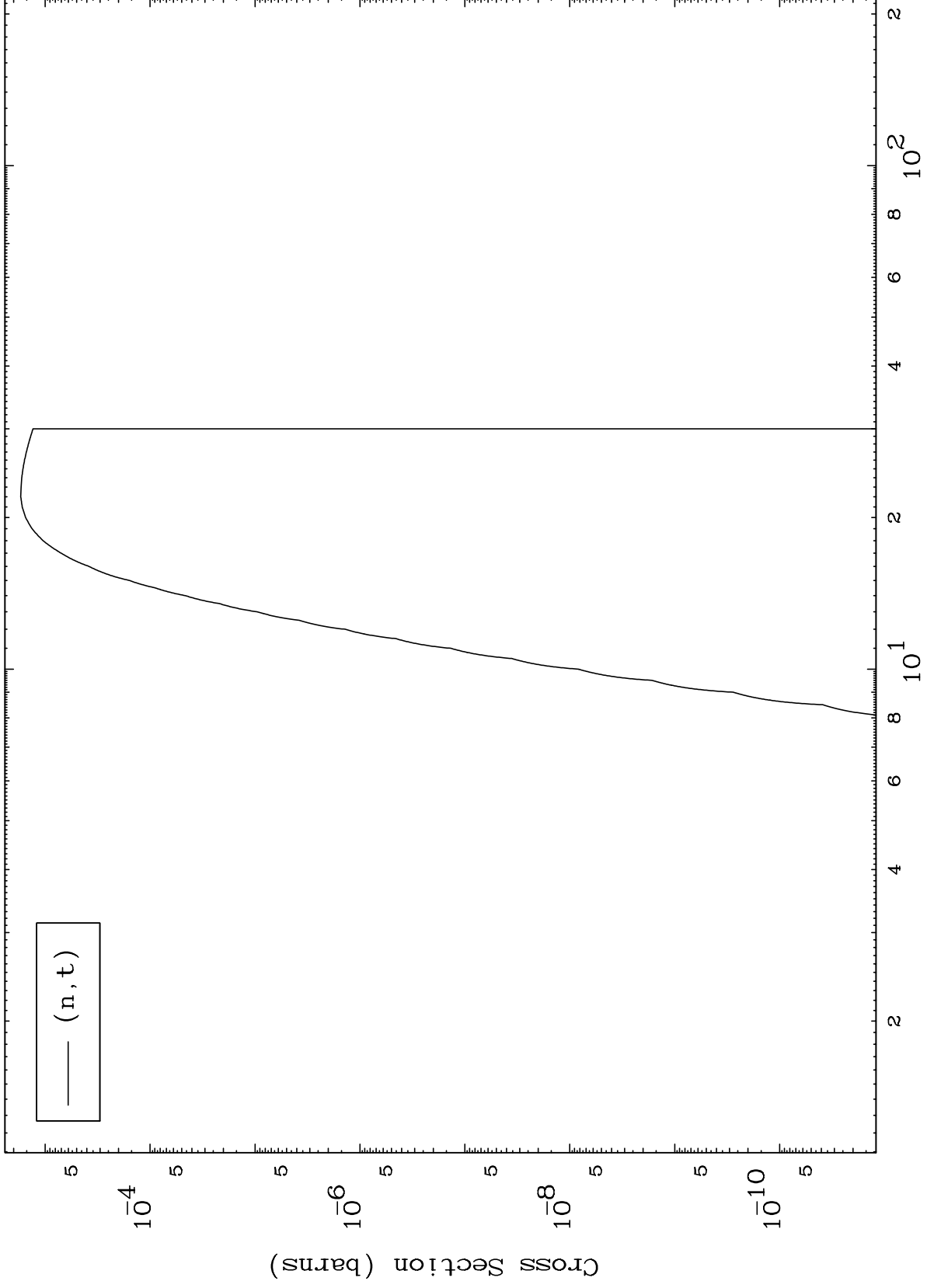
105-Db-269



MAT 569

(n,t) Levels  
293 Kelvin Cross Sections

105-Db-269



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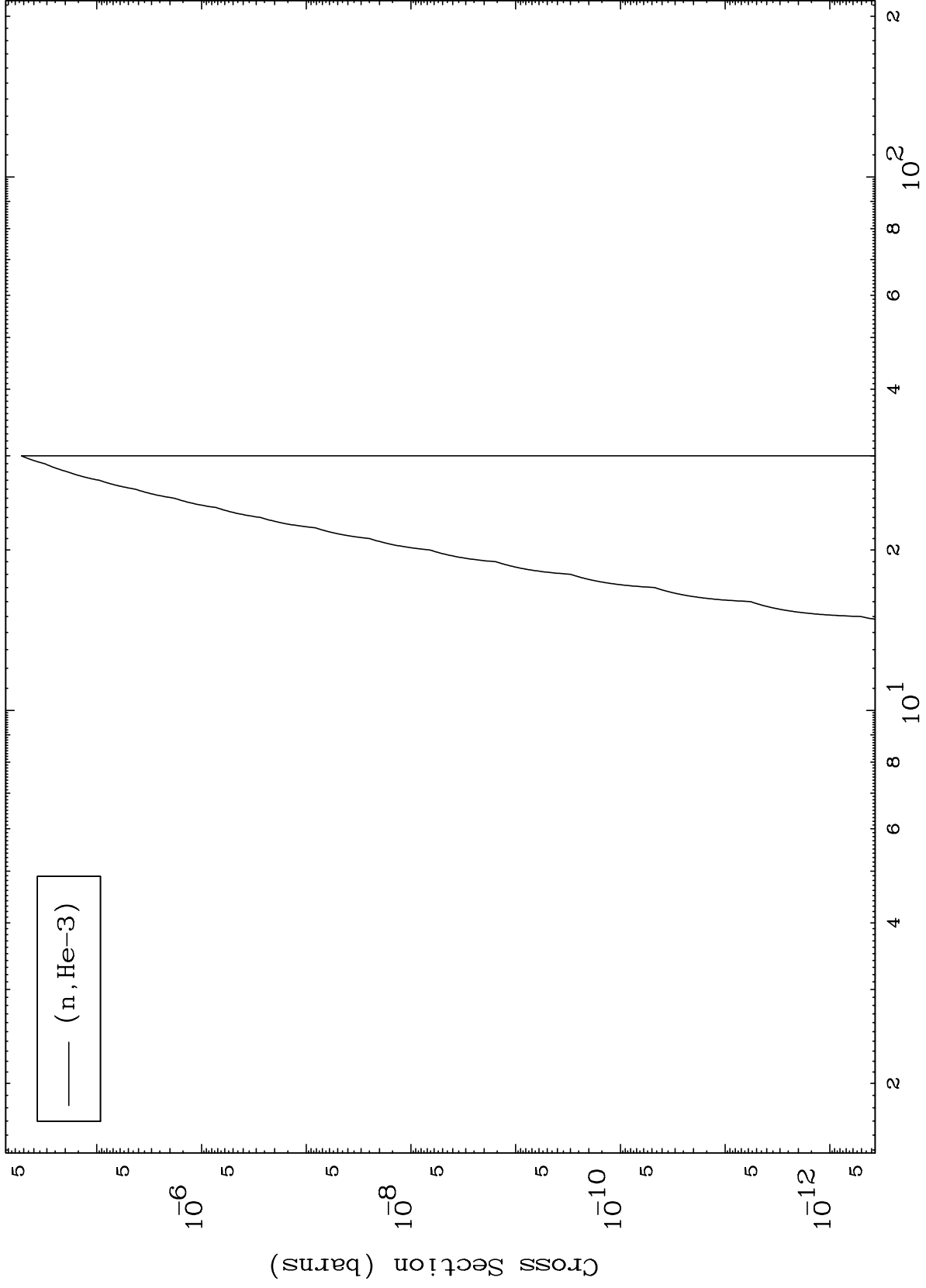
Incident Energy (MeV)

105-Db-269

MAT 569

(n,He3) Levels  
293 Kelvin Cross Sections

105-Db-269



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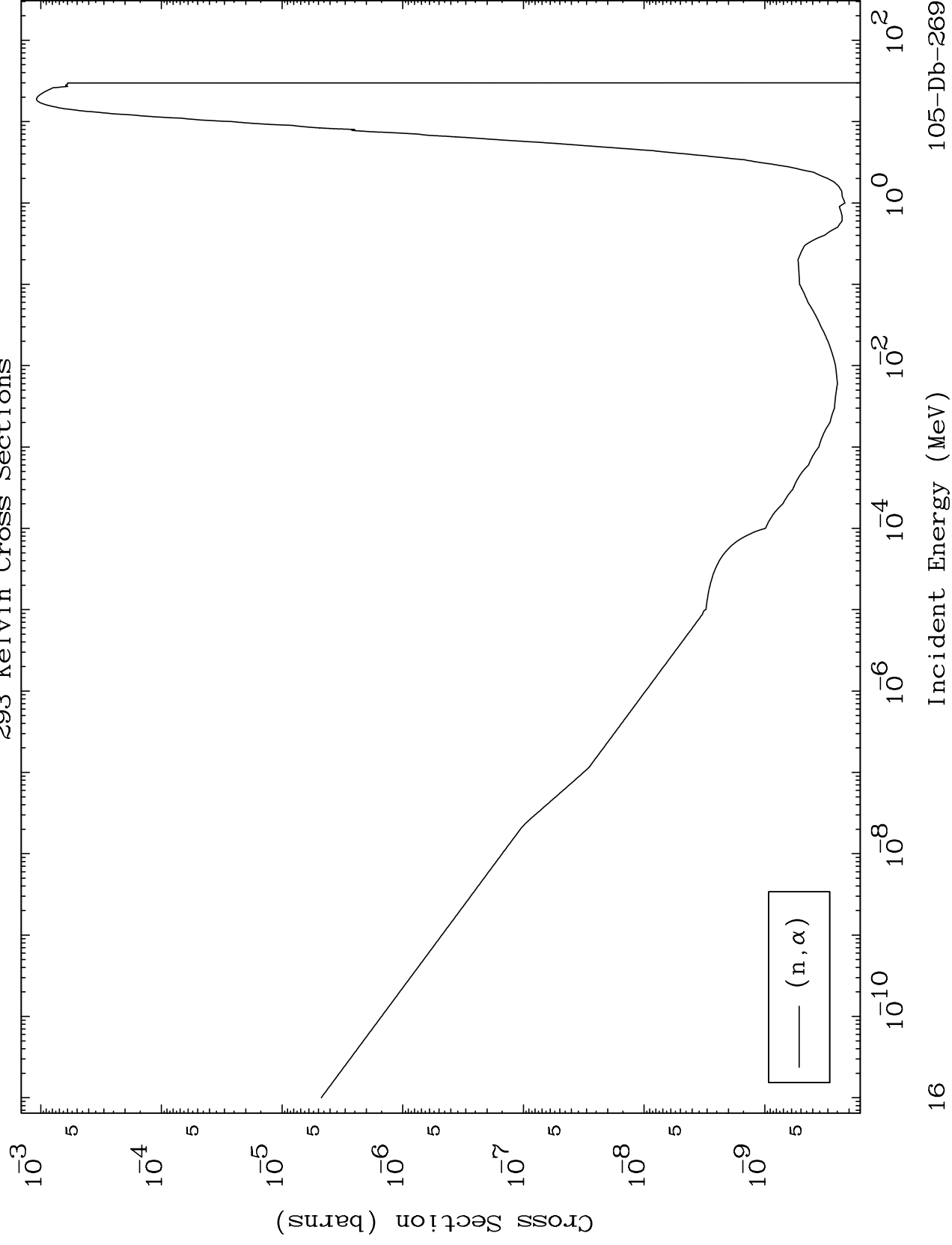
Incident Energy (MeV)

105-Db-269

MAT 569

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

105-Db-269

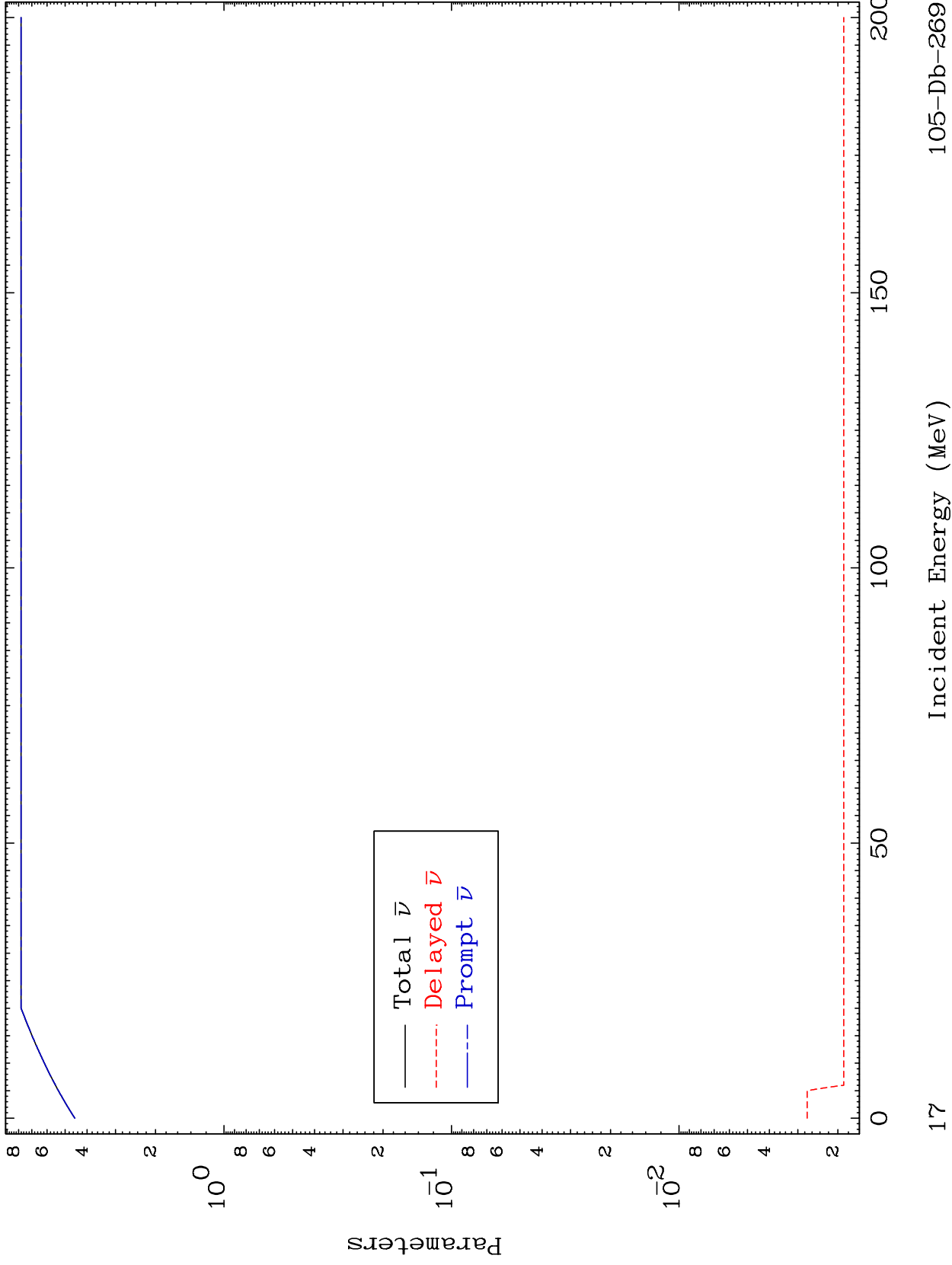


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Energy Release  
Parameters

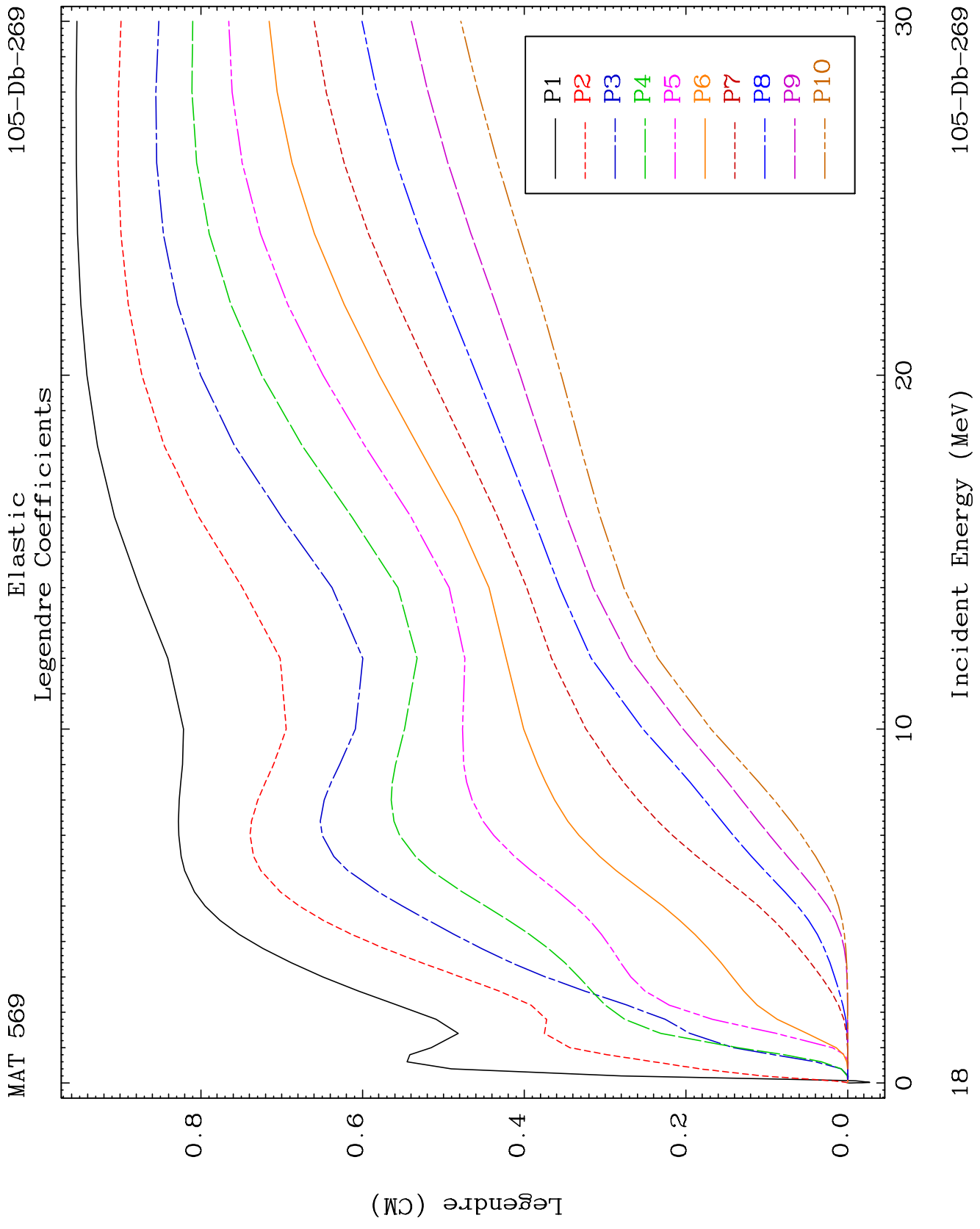
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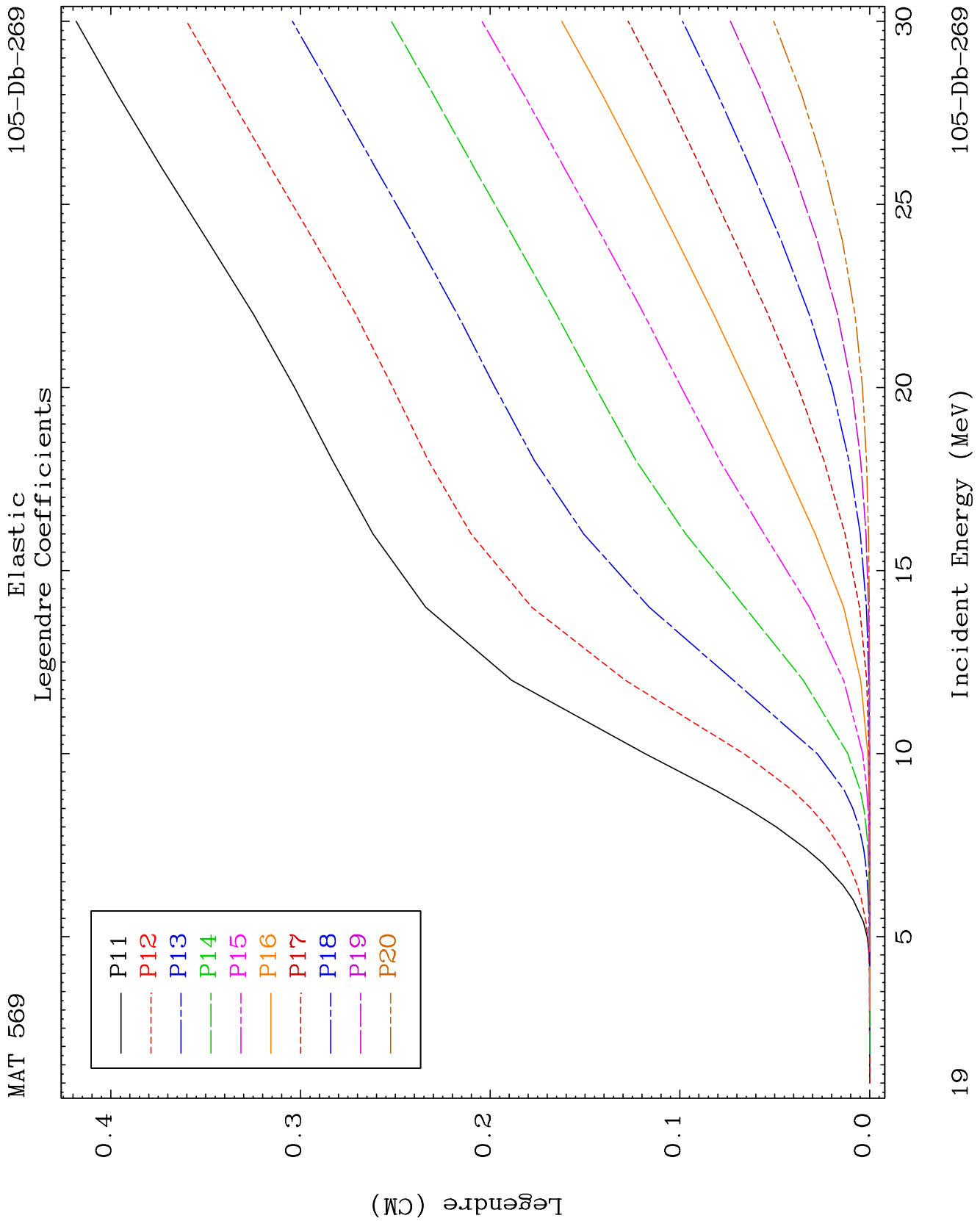


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Incident Energy (MeV)

105-Db-269

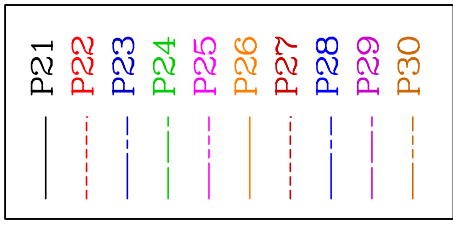
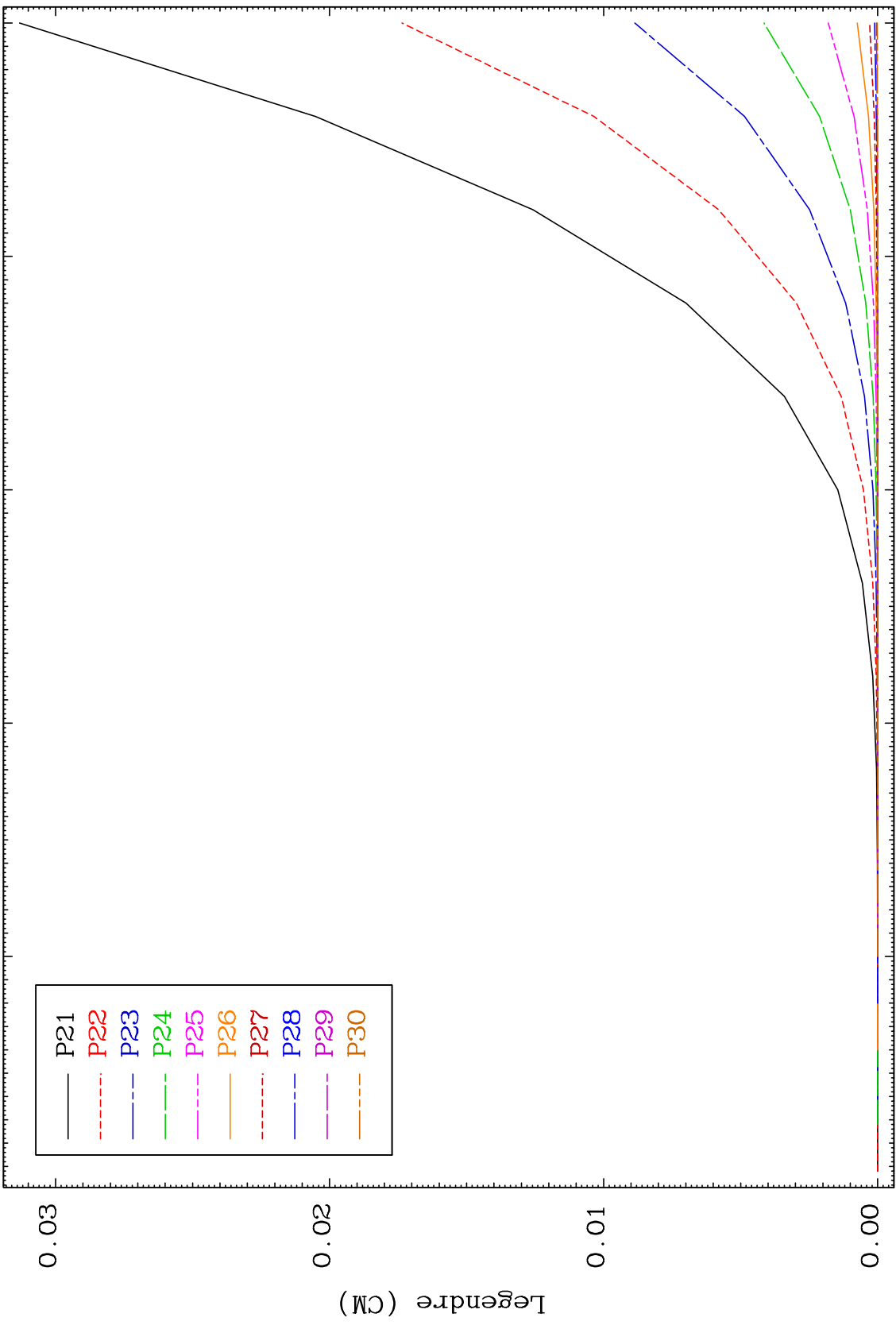




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Elastic Legendre Coefficients

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Incident Energy (MeV)

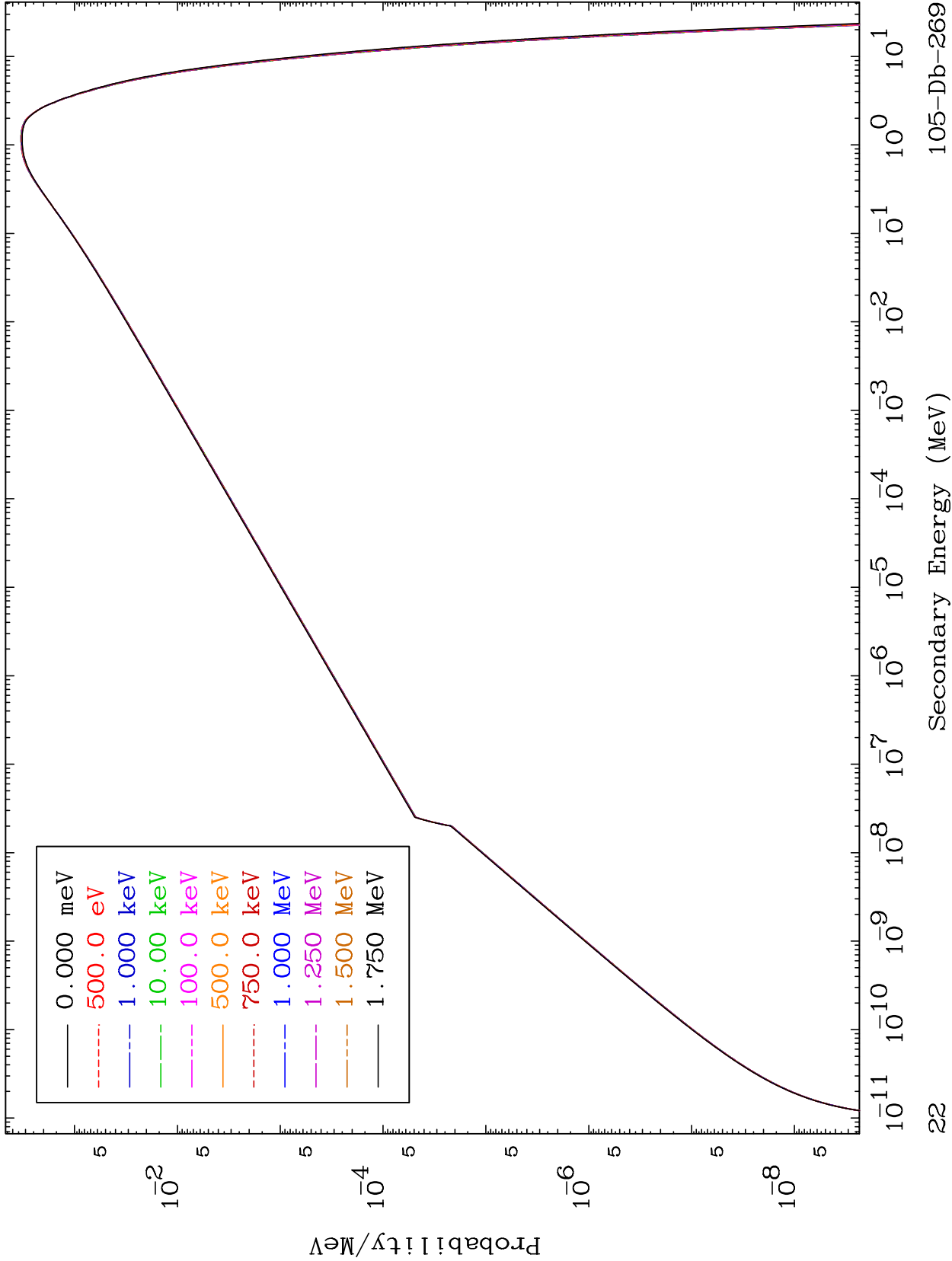
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### Fission Energy Distributions

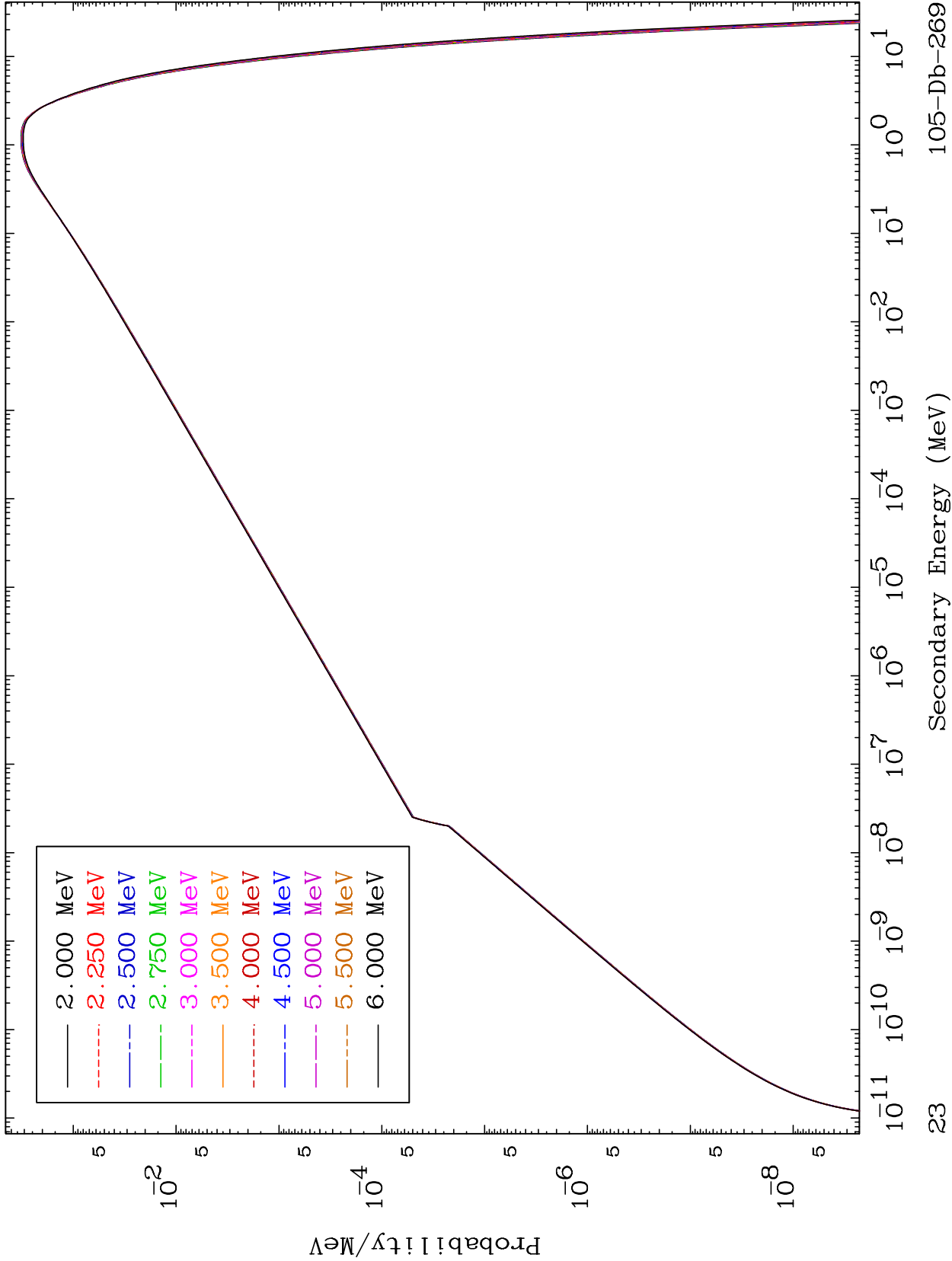
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### Fission Energy Distributions

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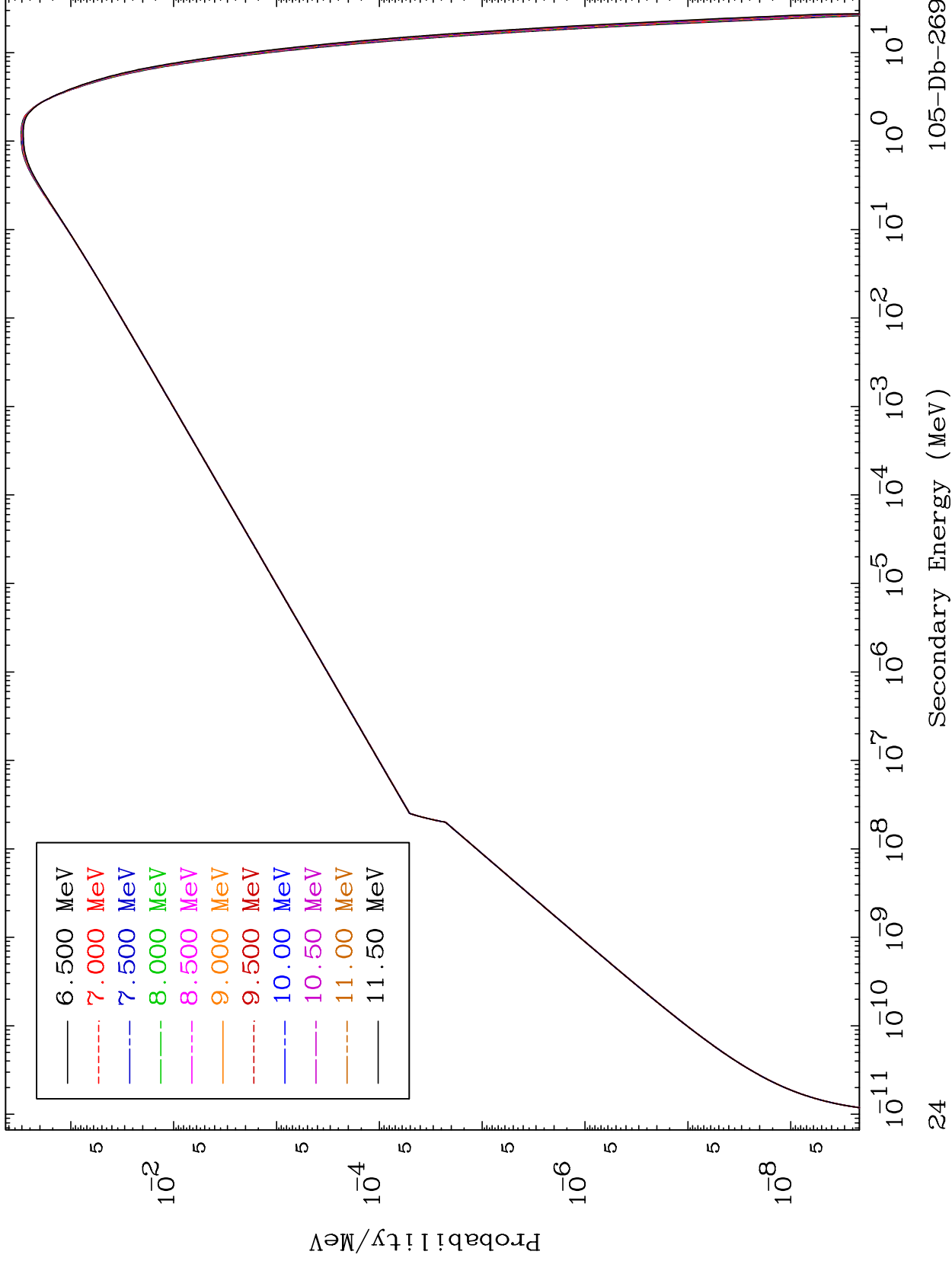
105-Db-269

Secondary Energy (MeV)

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### Fission Energy Distributions

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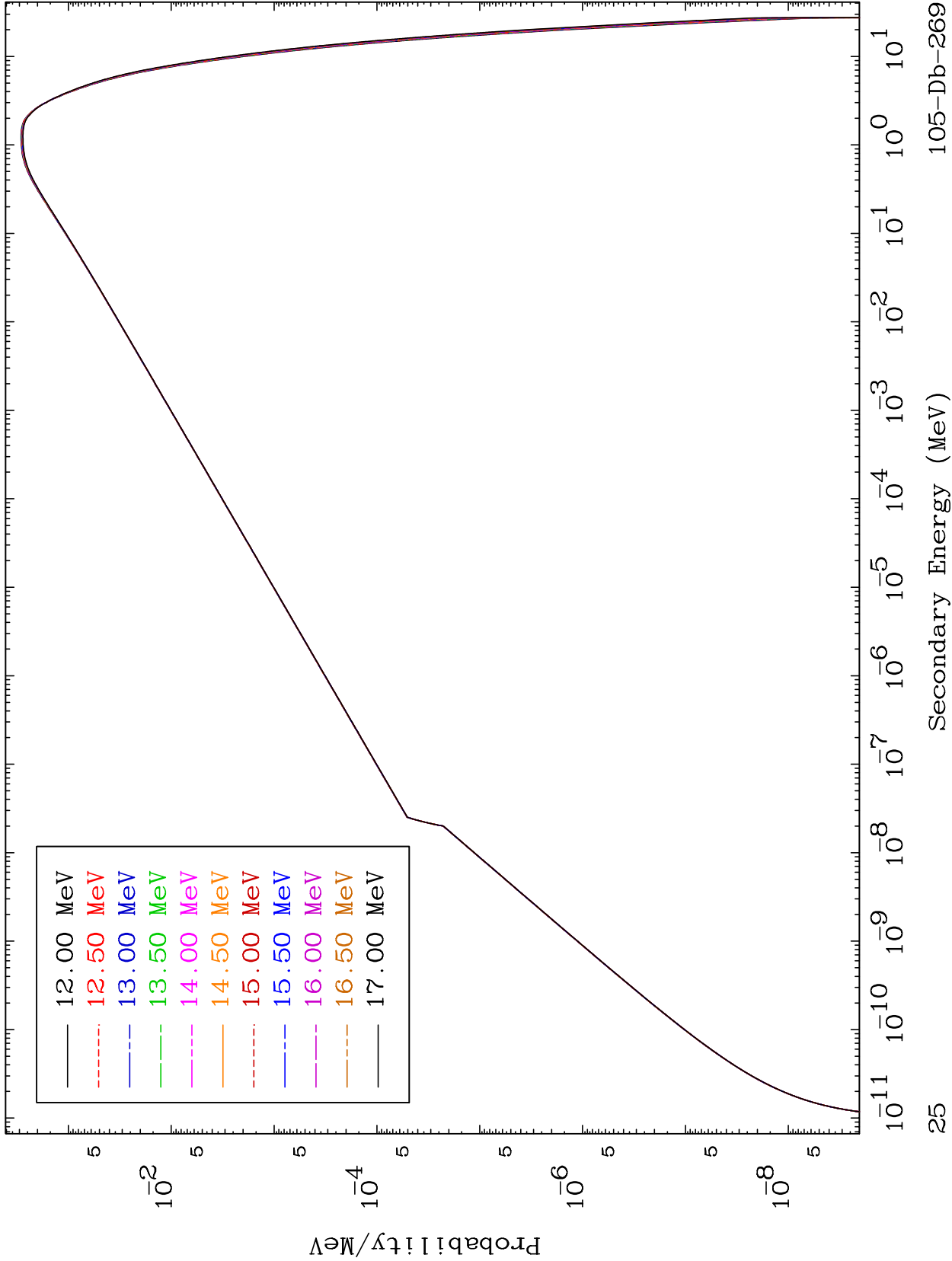
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105-Db-269

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### Fission Energy Distributions

105-Db-269



105-Db-269

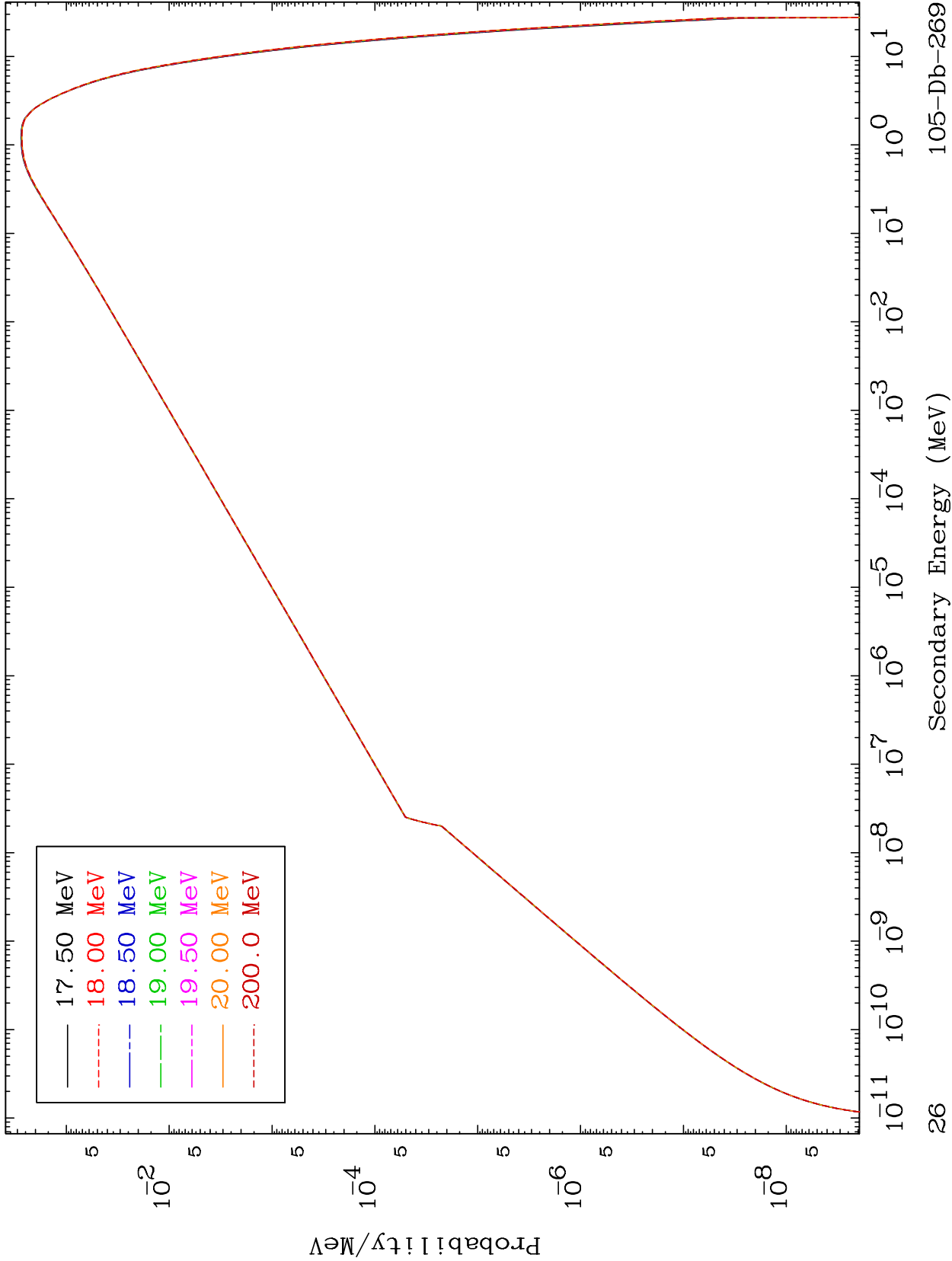
Secondary Energy (MeV)

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### Fission Energy Distributions

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Secondary Energy (MeV)

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