

Program EVALPLOT  
(Version 2021-1)

by

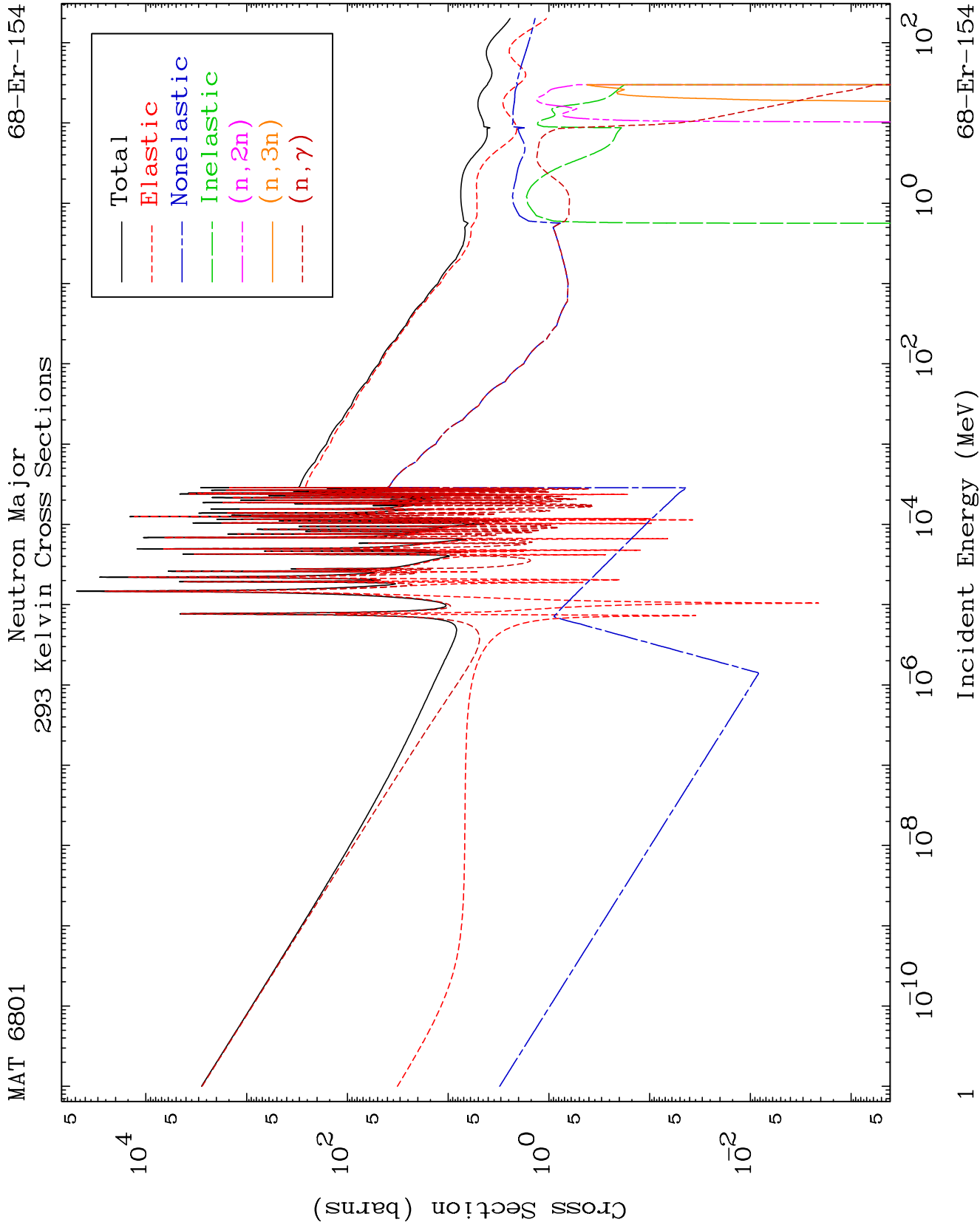
Dermott E. Cullen  
(Present Contact Information)

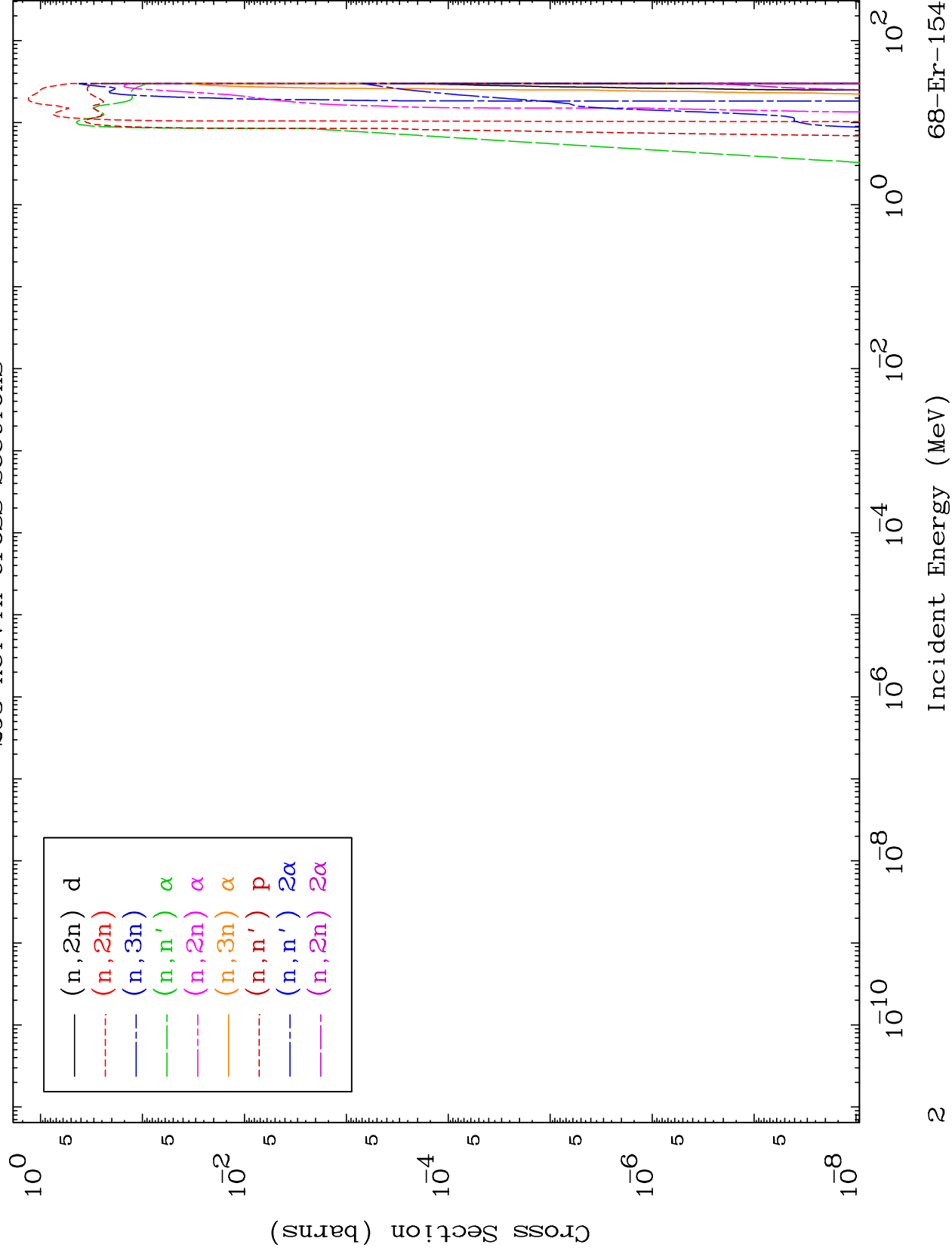
Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

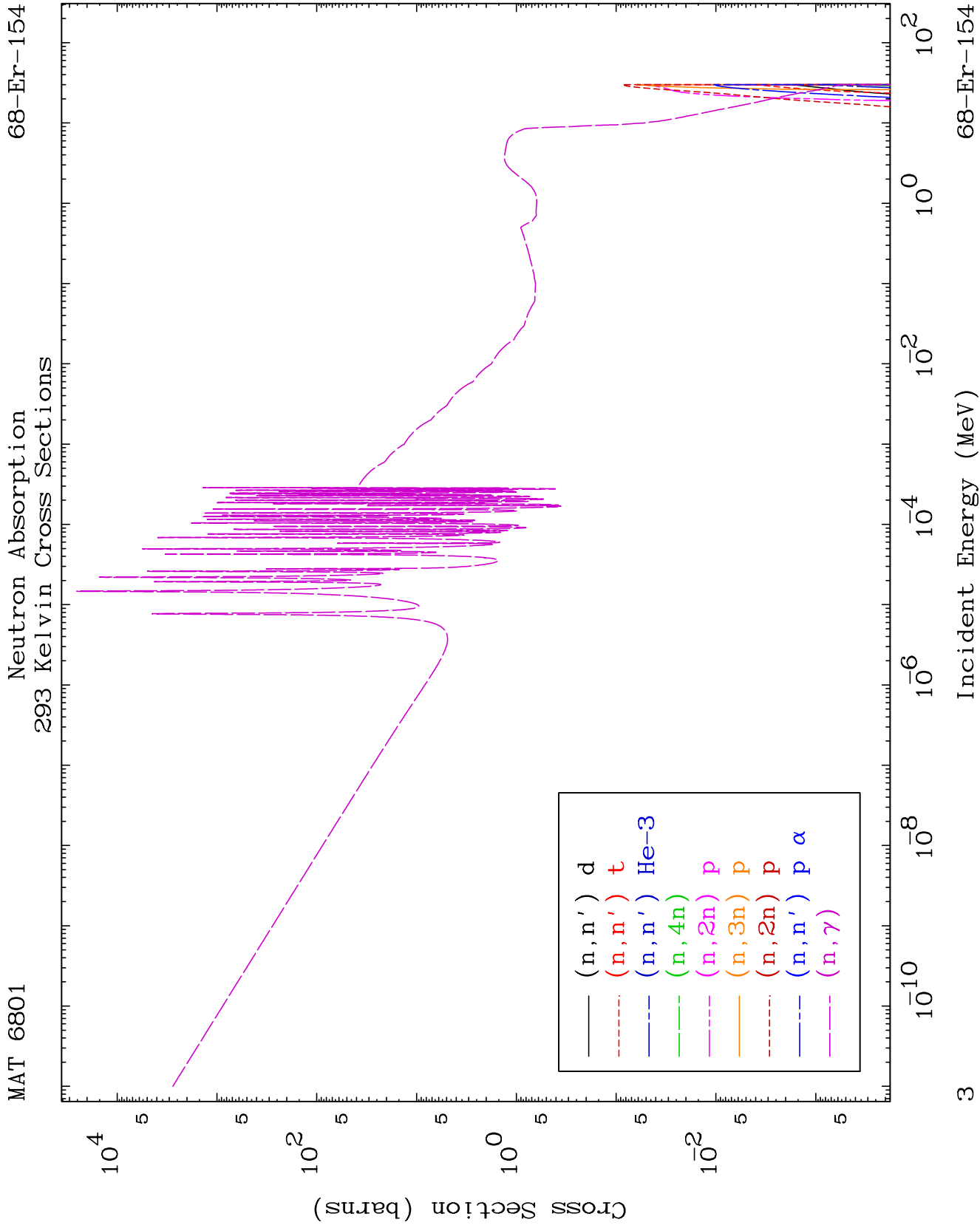
Tele: 925-443-1911

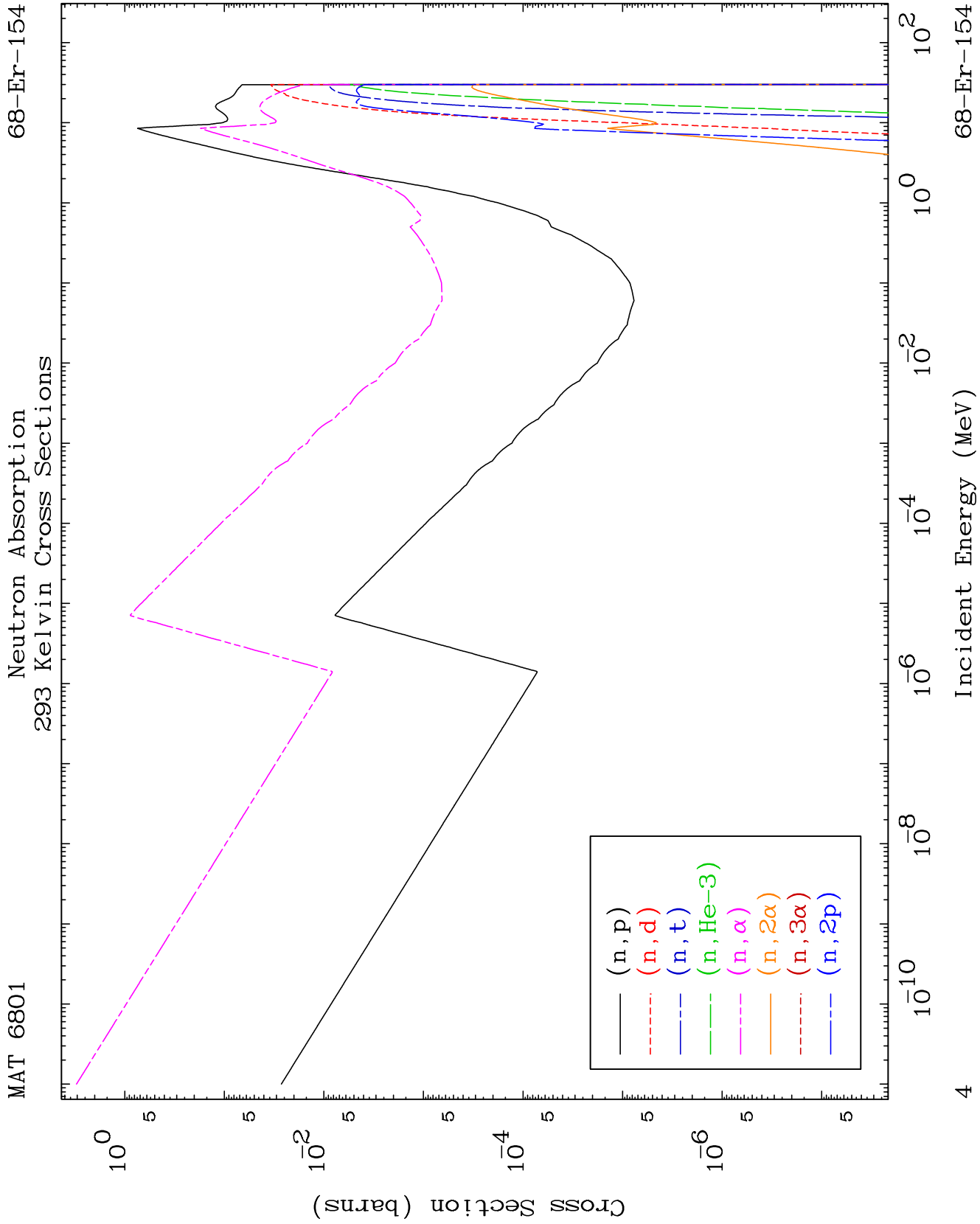
E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)  
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start





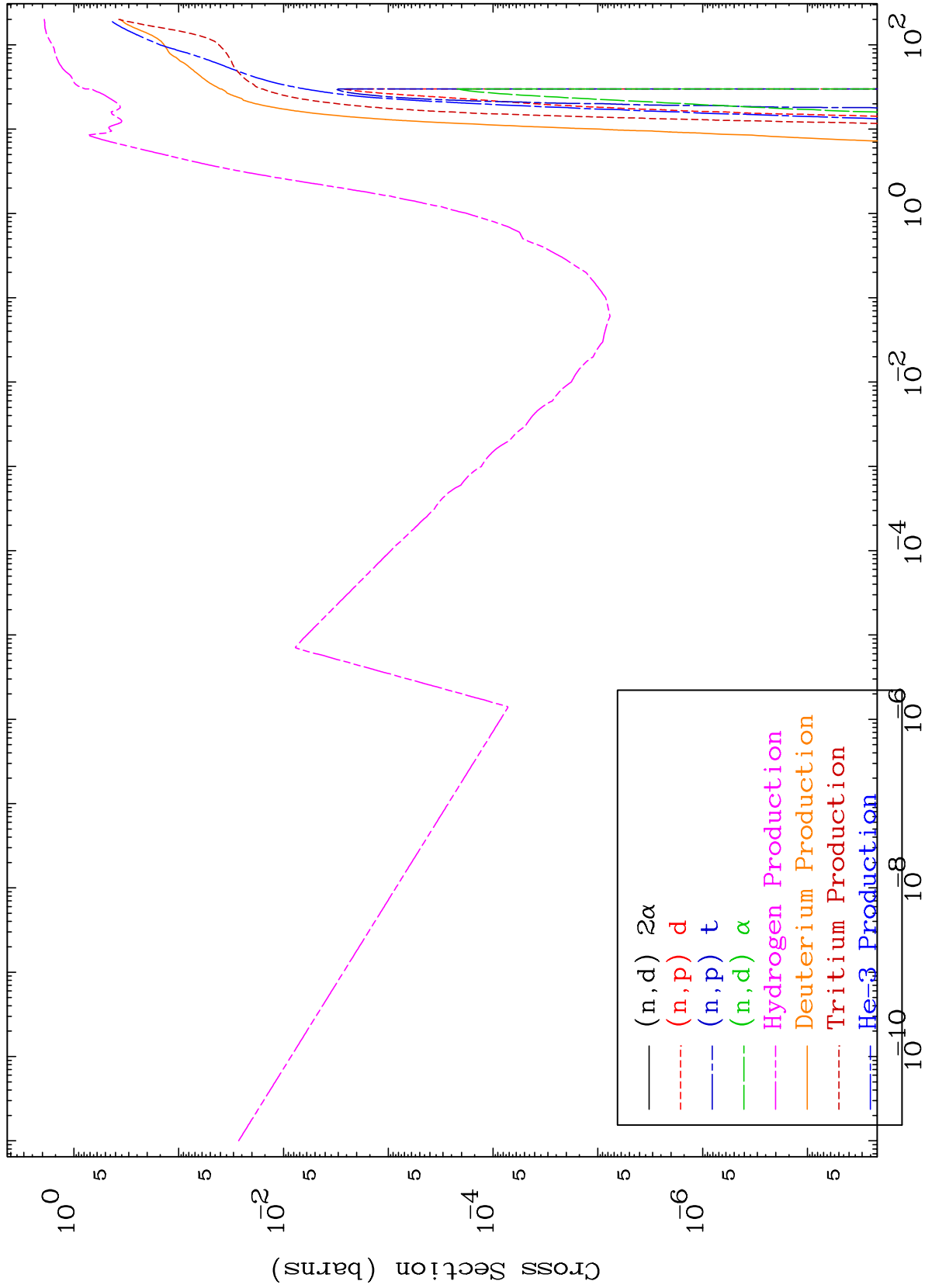


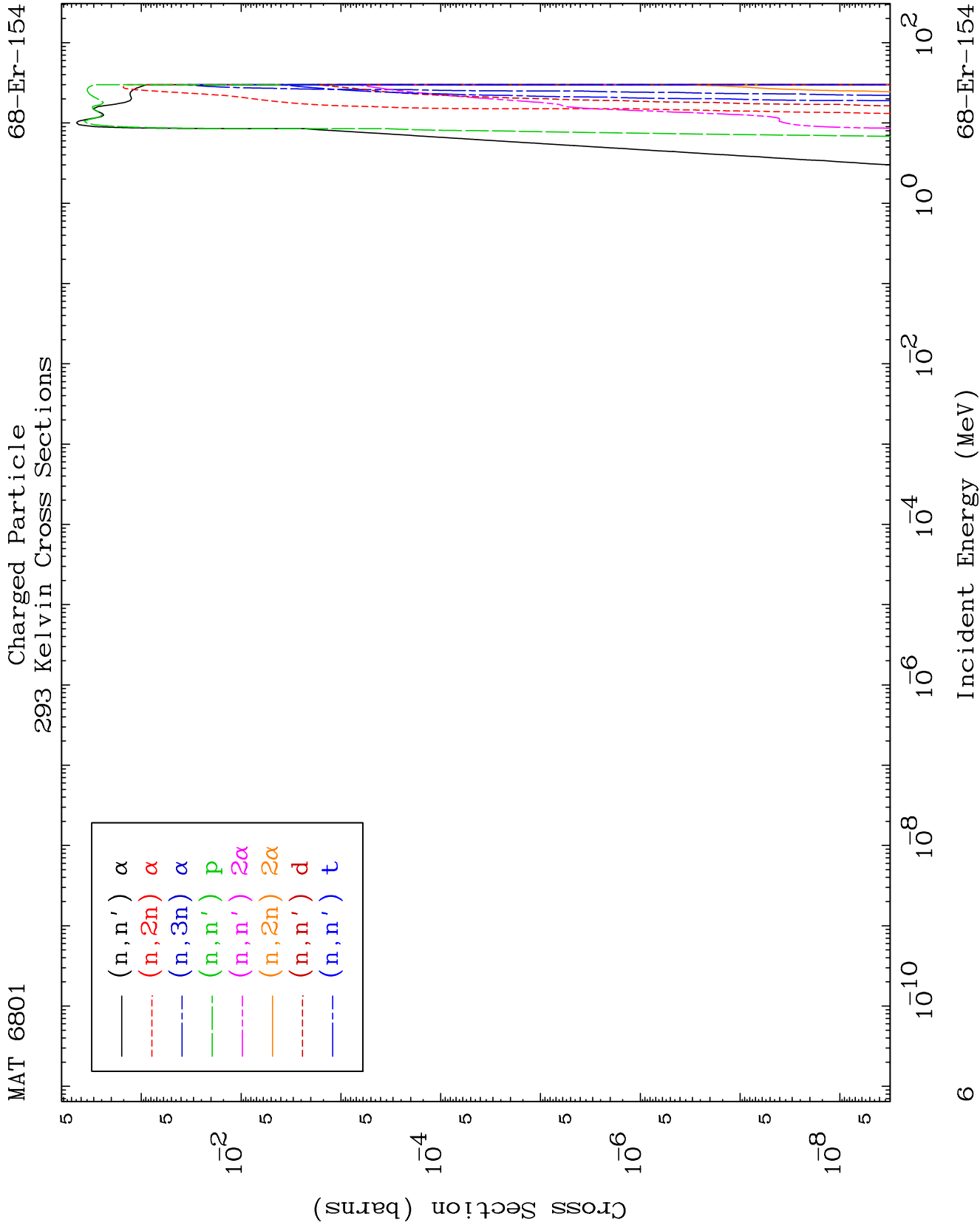


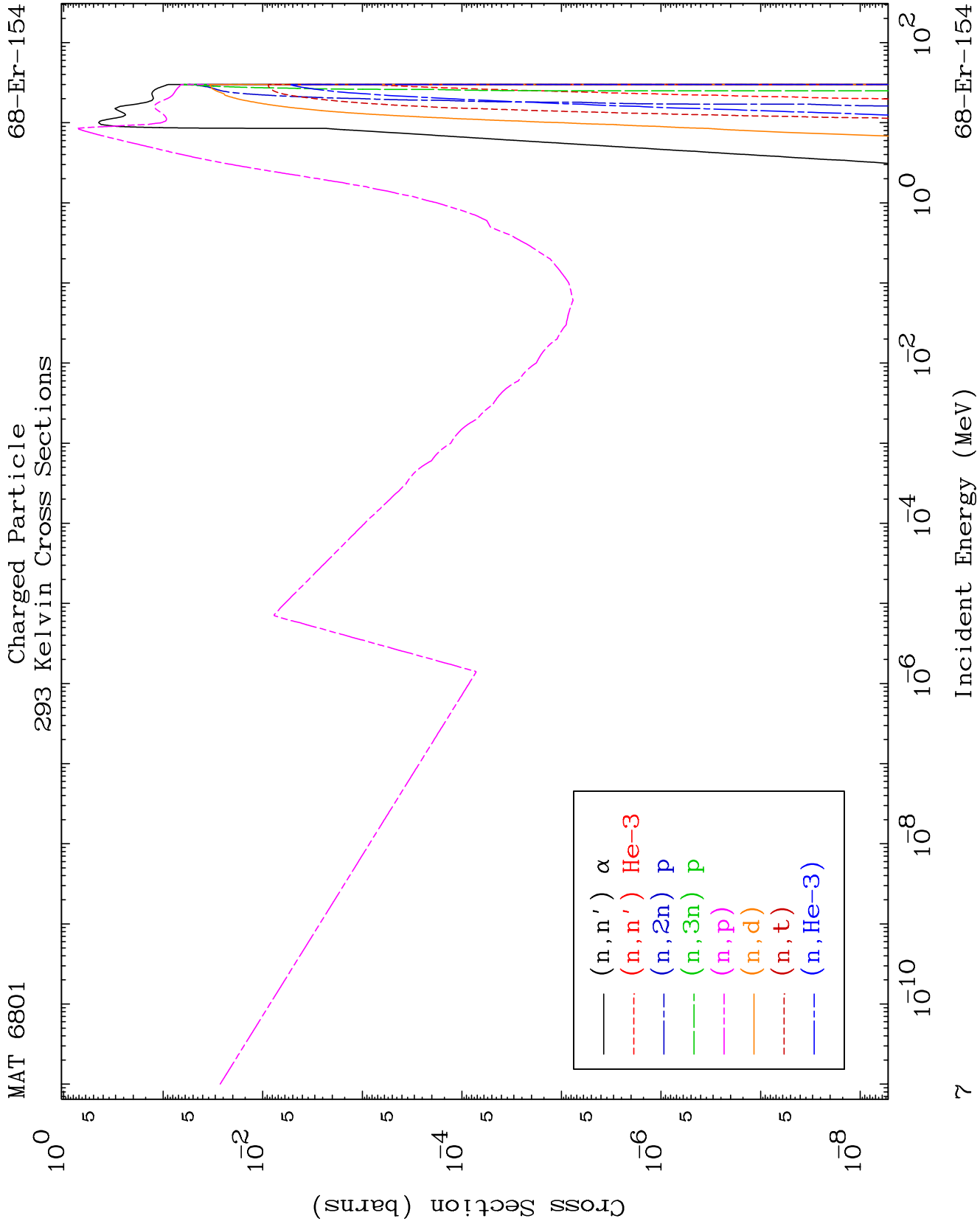
MAT 6801

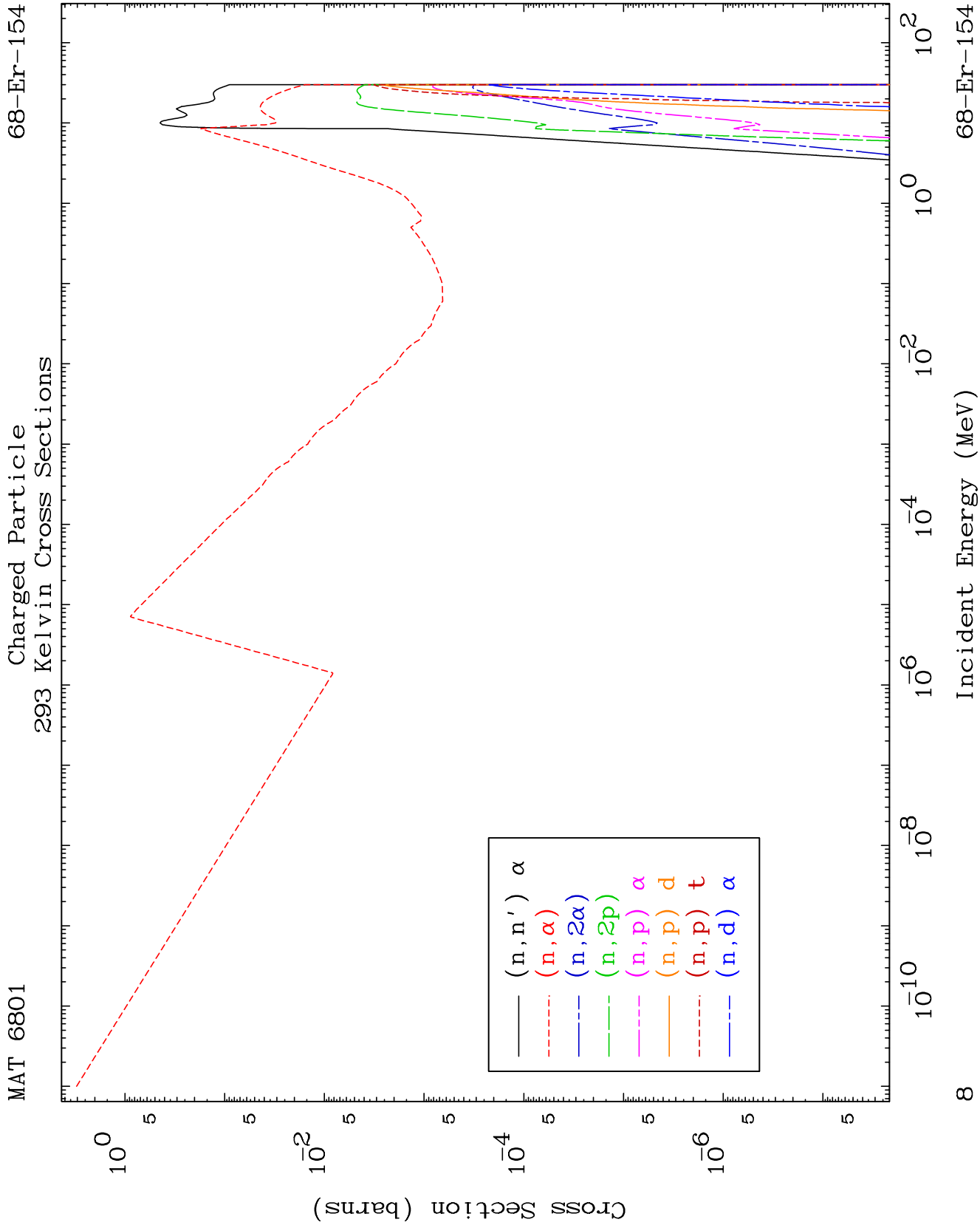
Neutron Absorption  
293 Kelvin Cross Sections

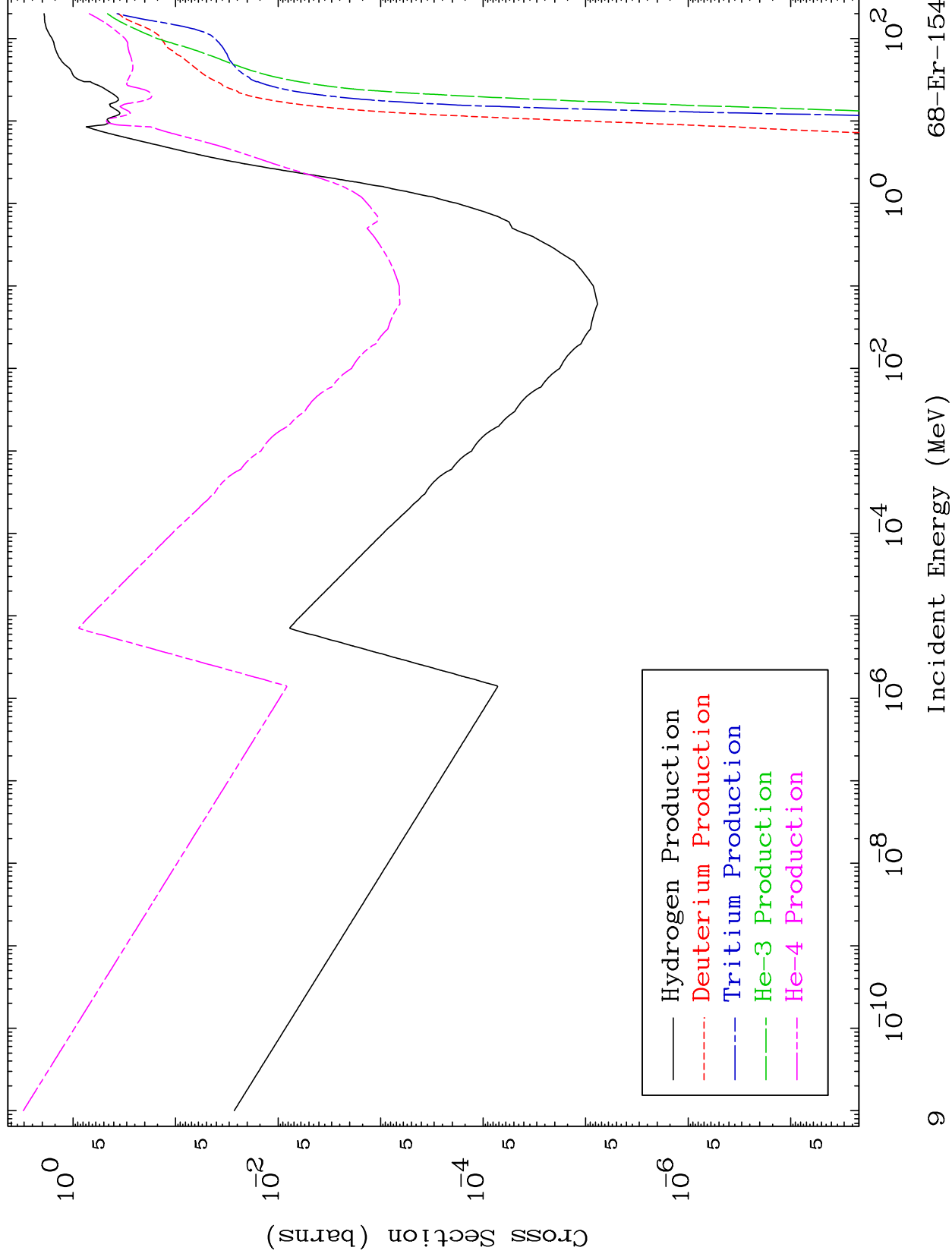
68-Er-154

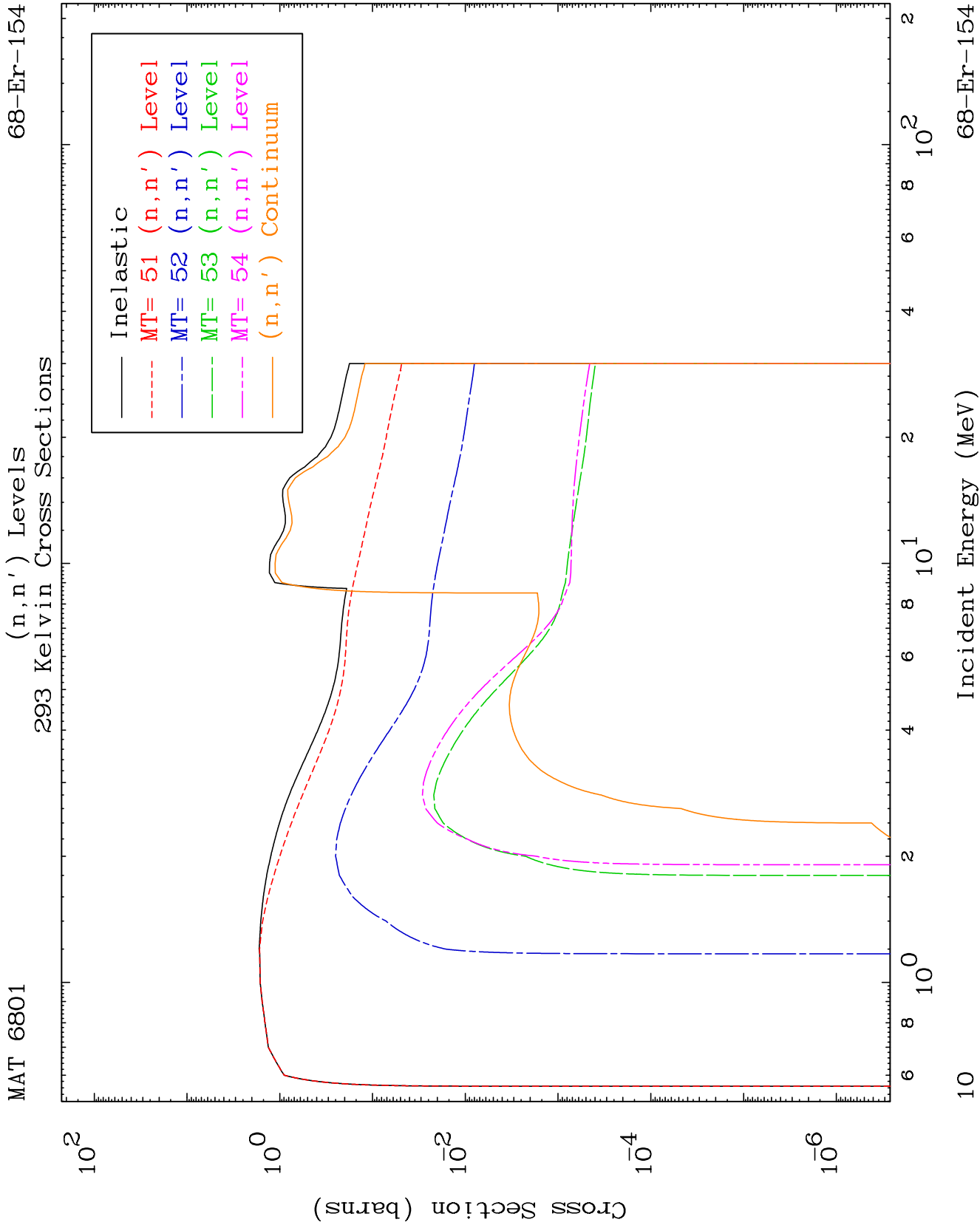








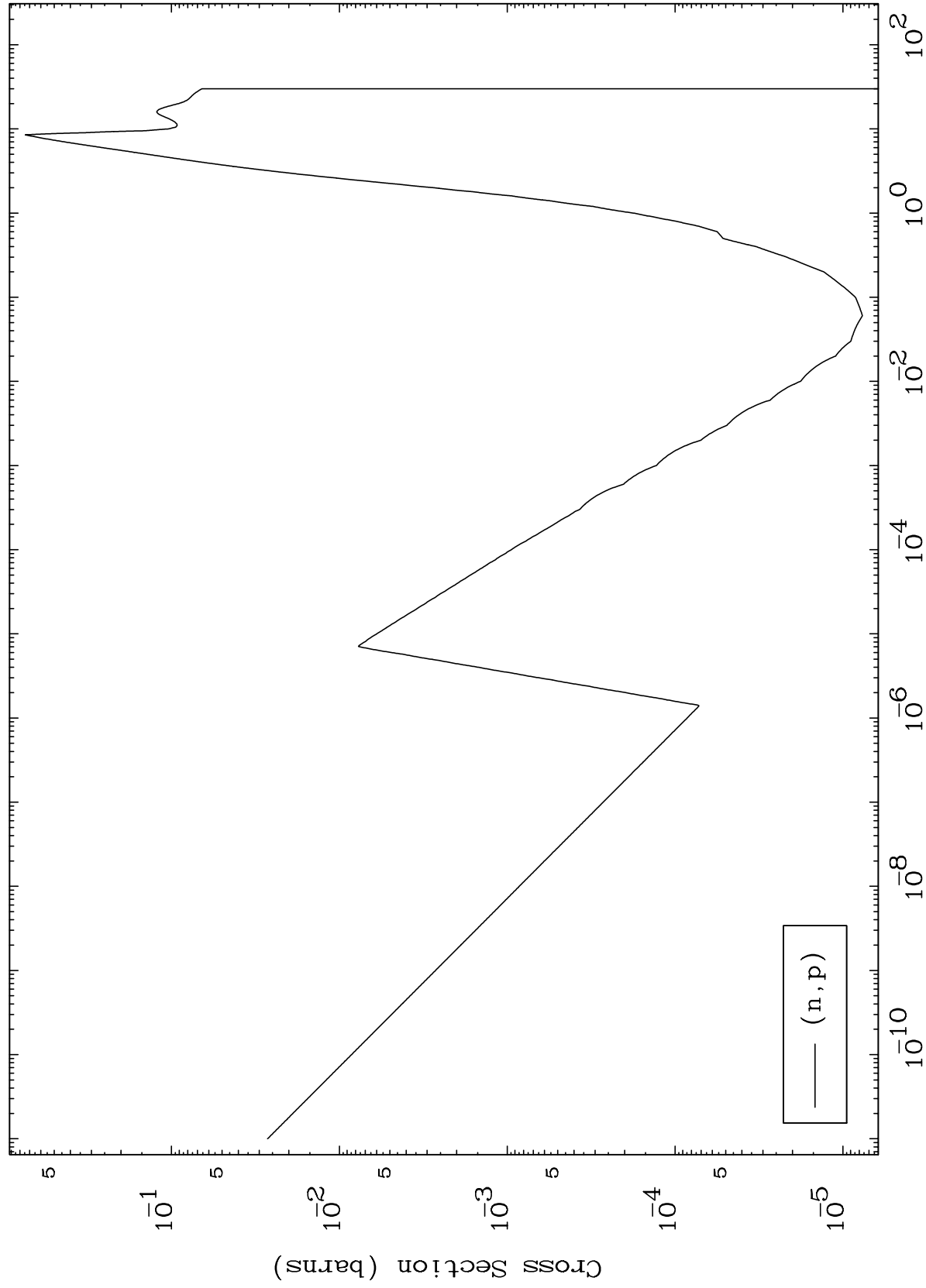




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(n,p) Levels  
293 Kelvin Cross Sections

68-Er-154



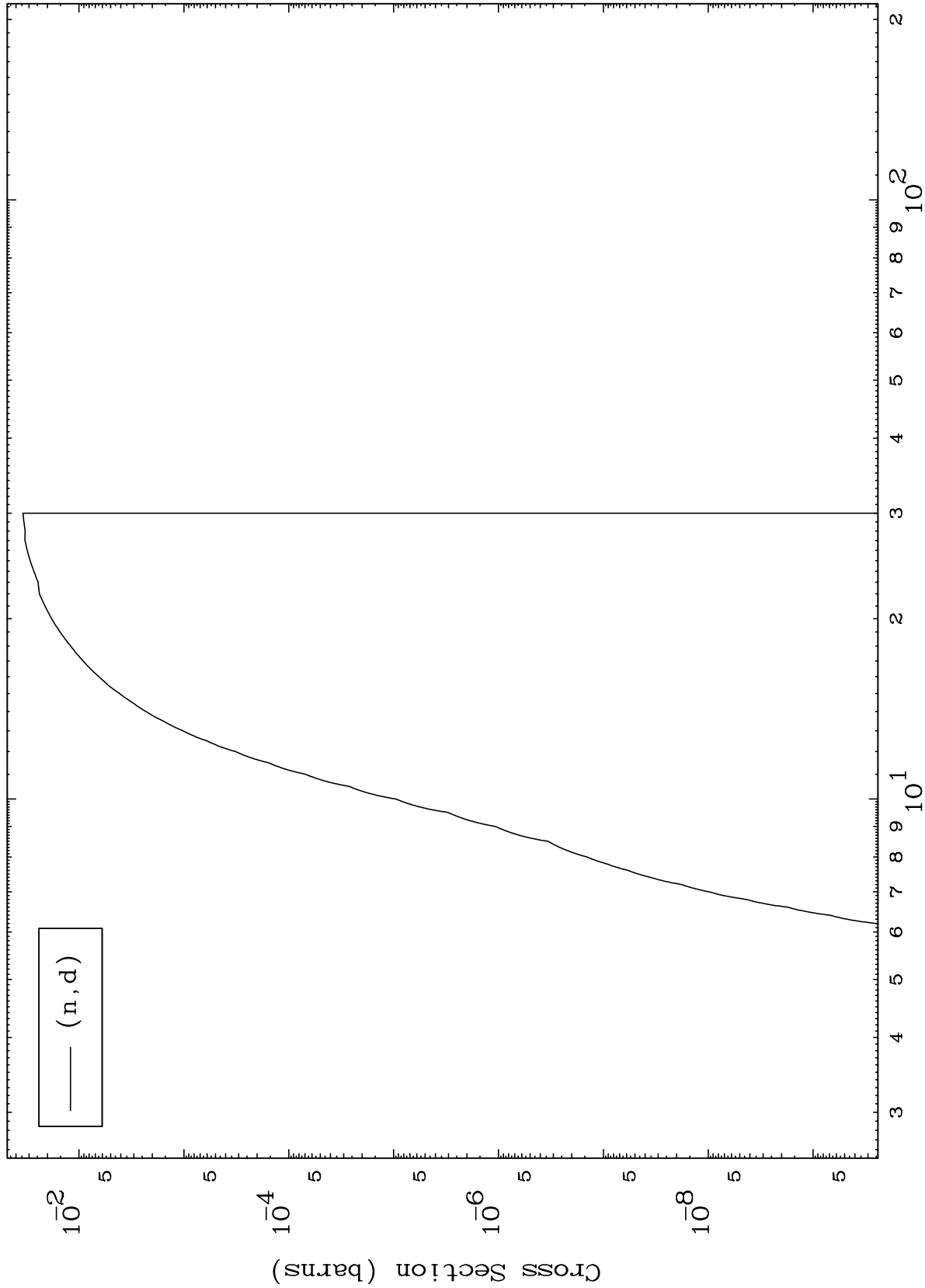
11

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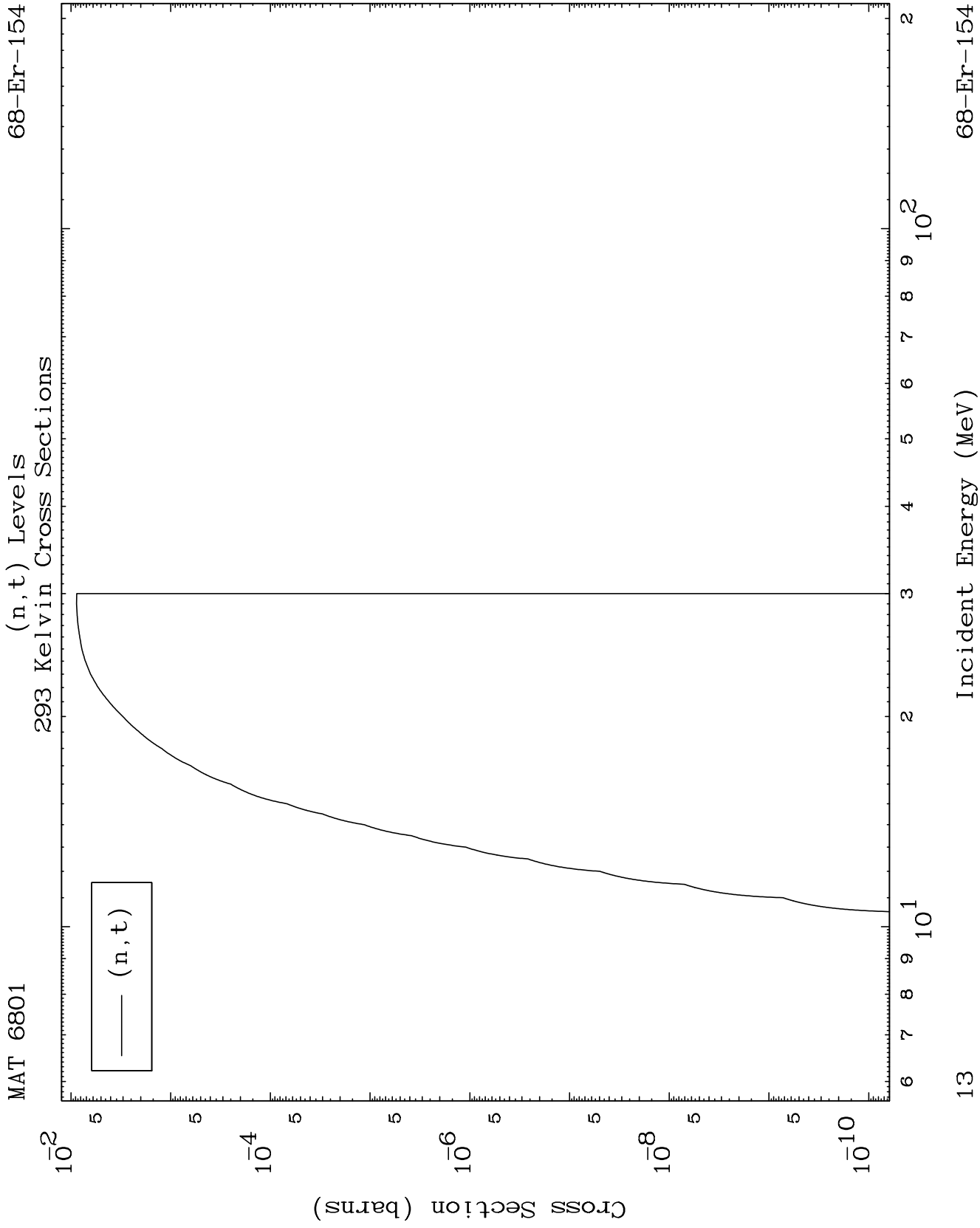
(n,d) Levels  
293 Kelvin Cross Sections

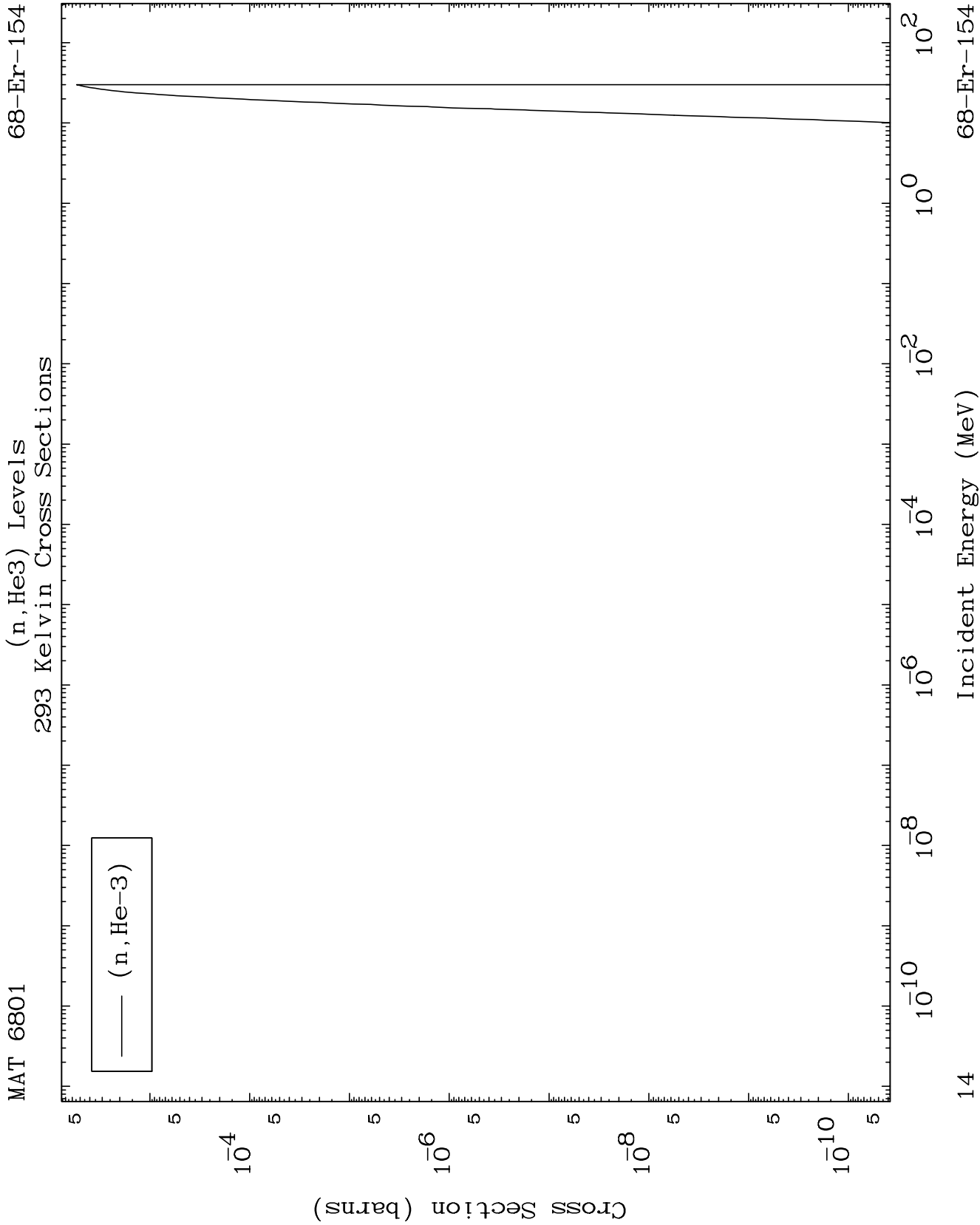


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Incident Energy (MeV)

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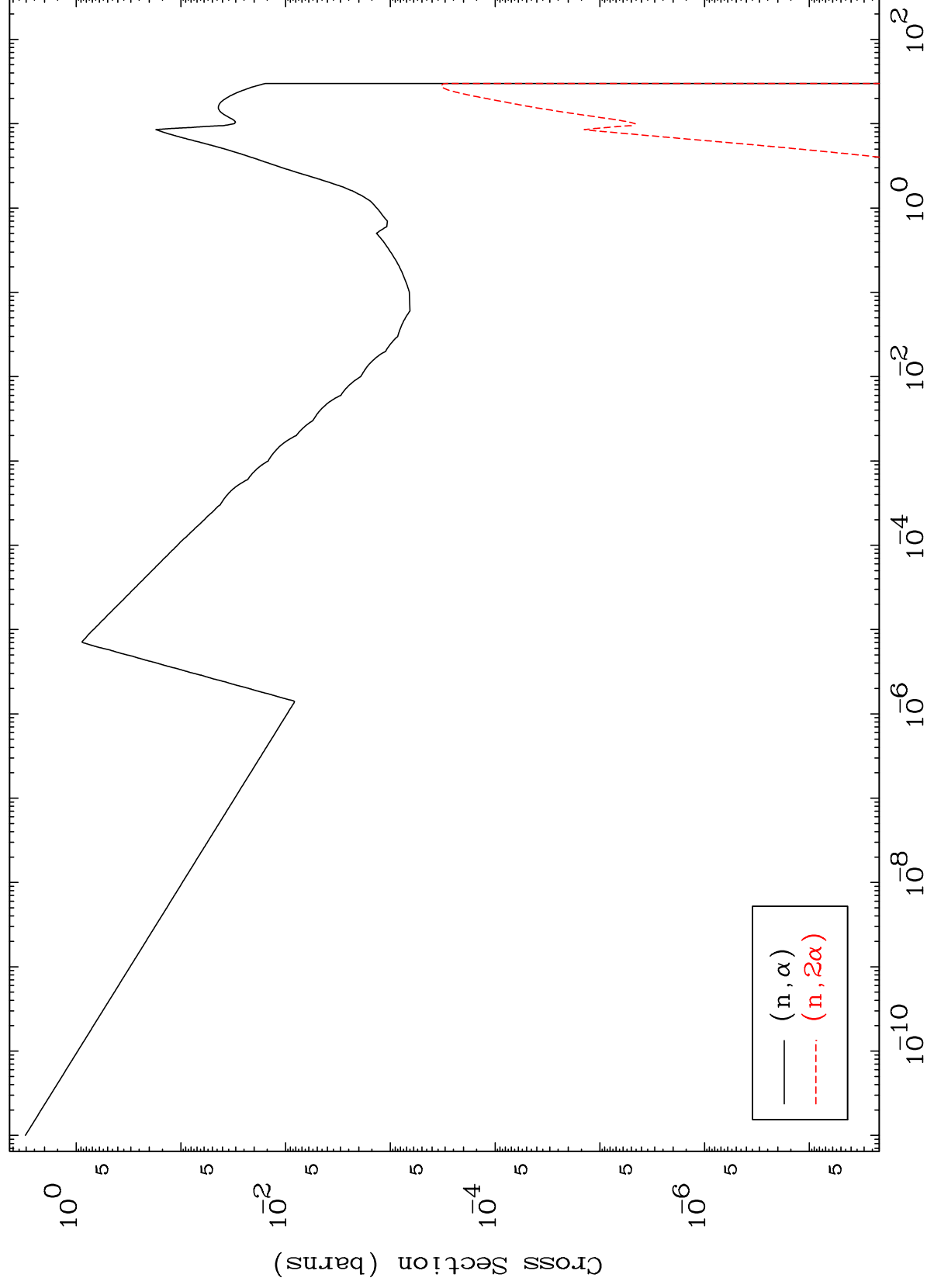




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68-Er-154

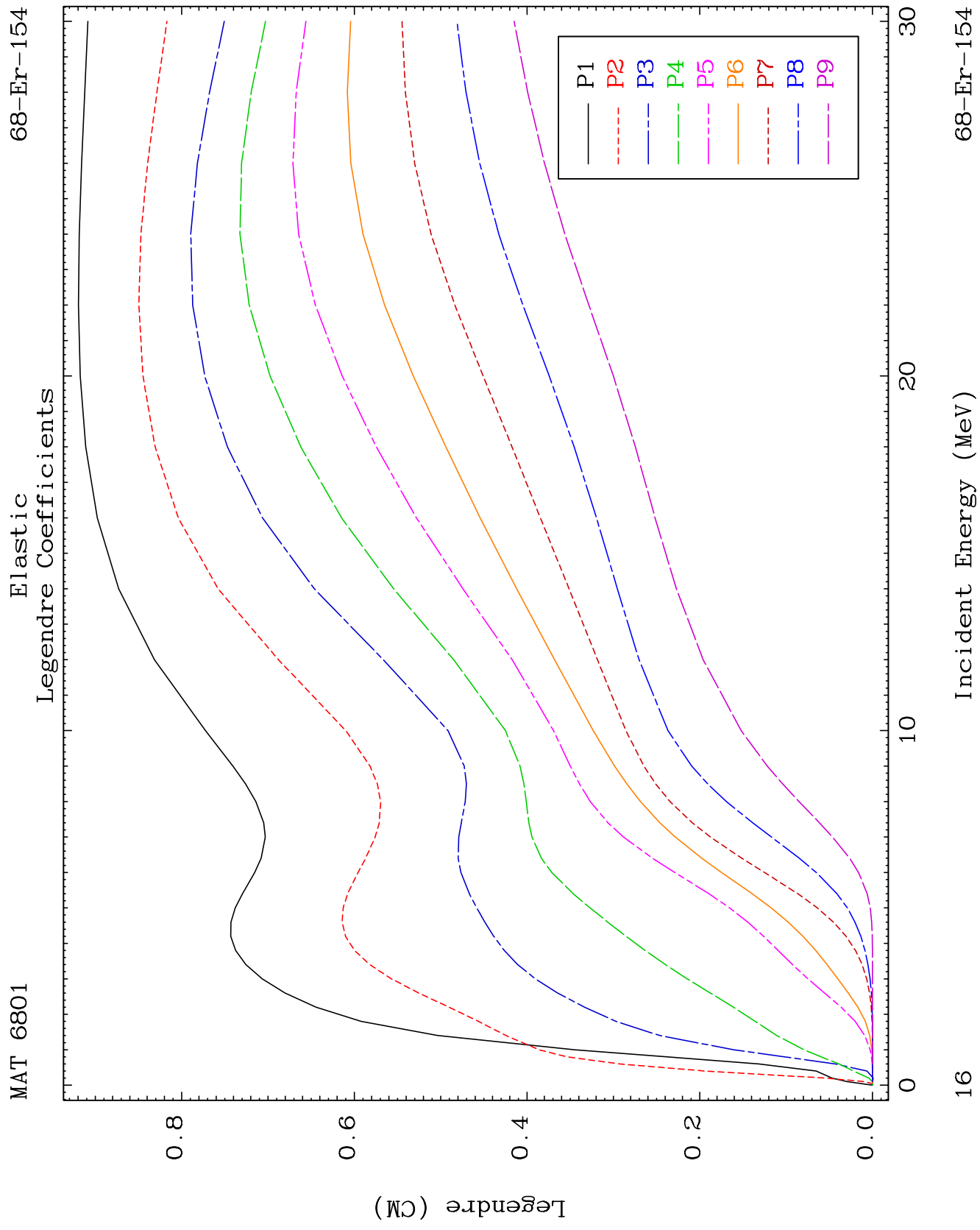
(n, $\alpha$ ) Levels  
293 Kelvin Cross Sections

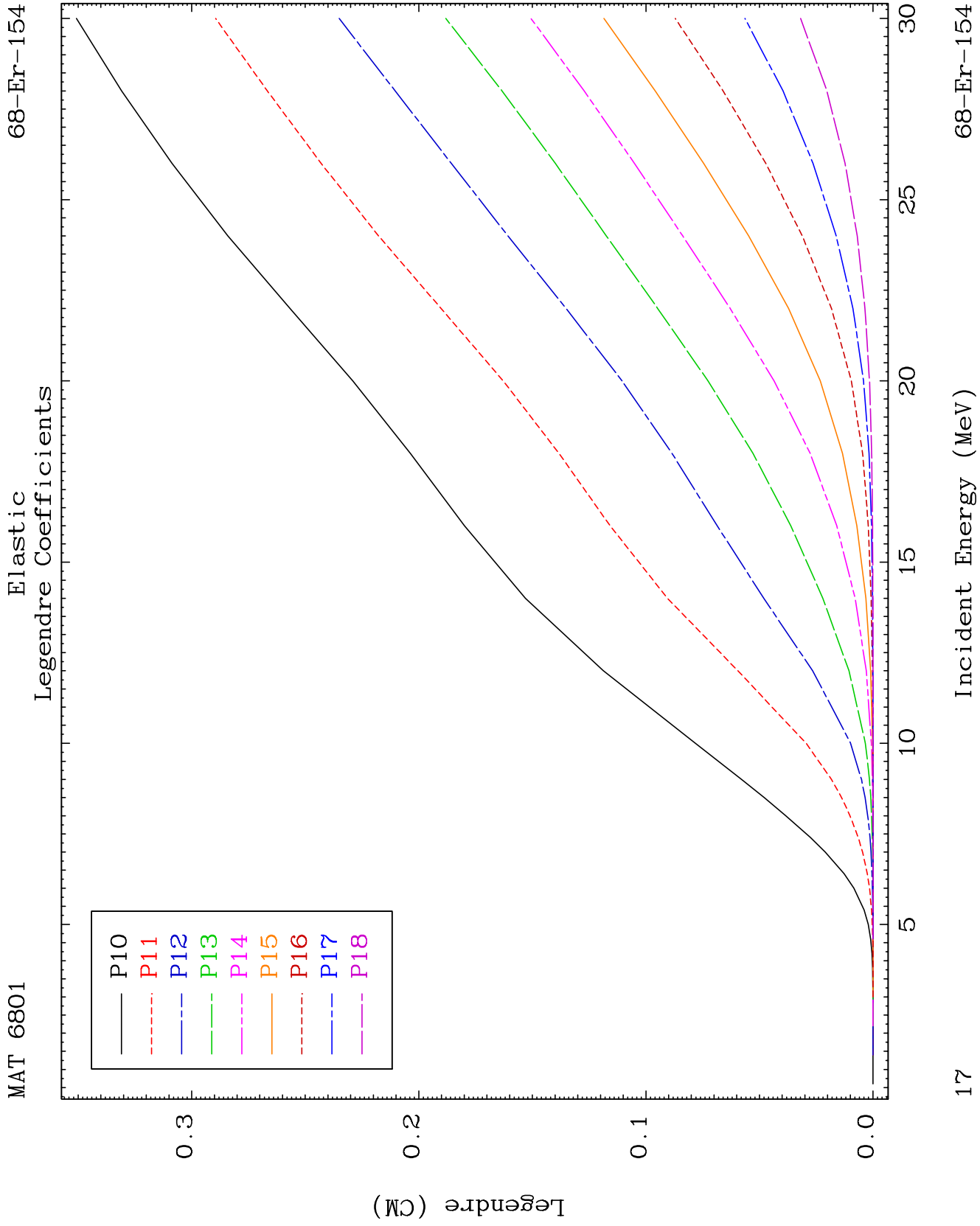


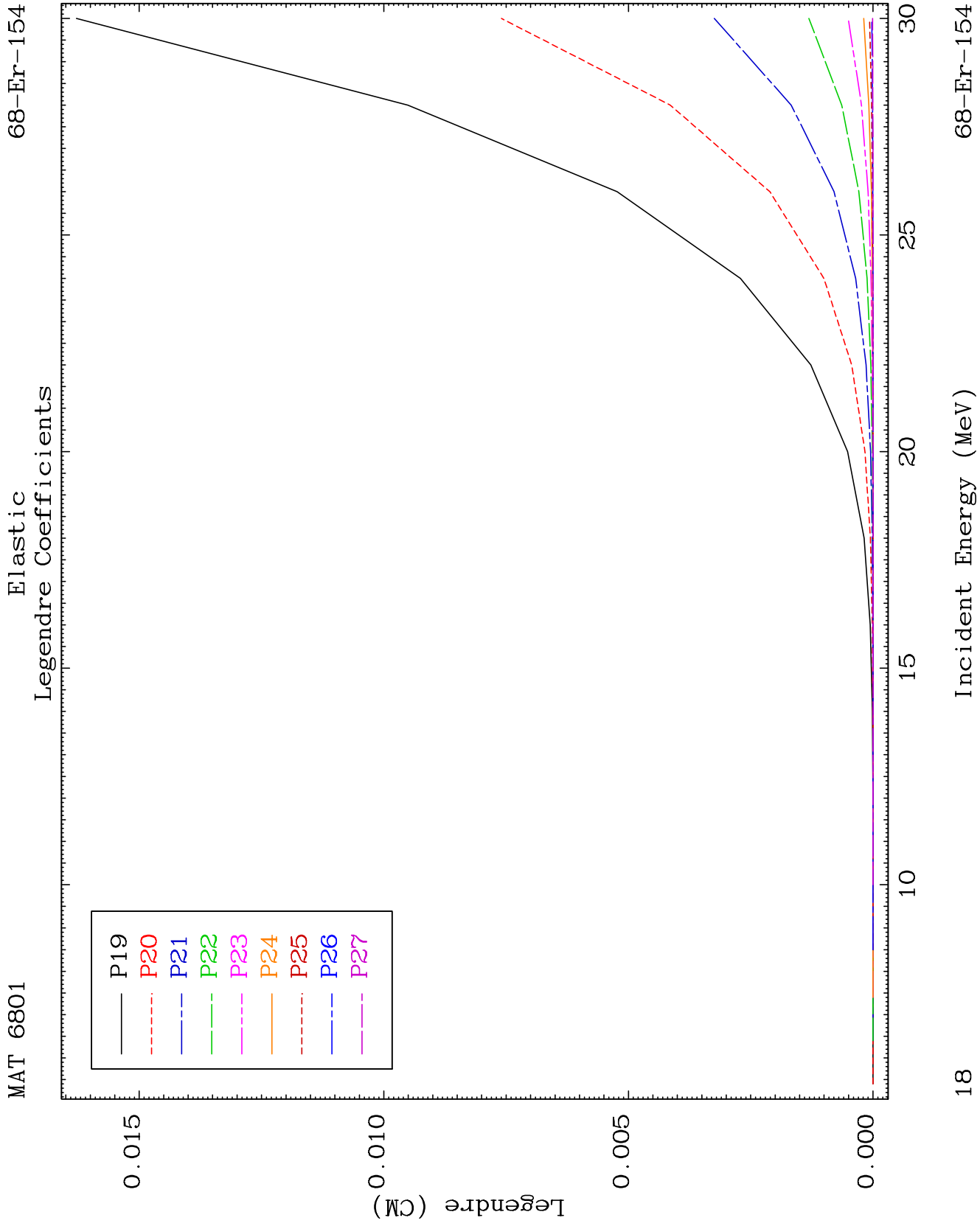
68-Er-154

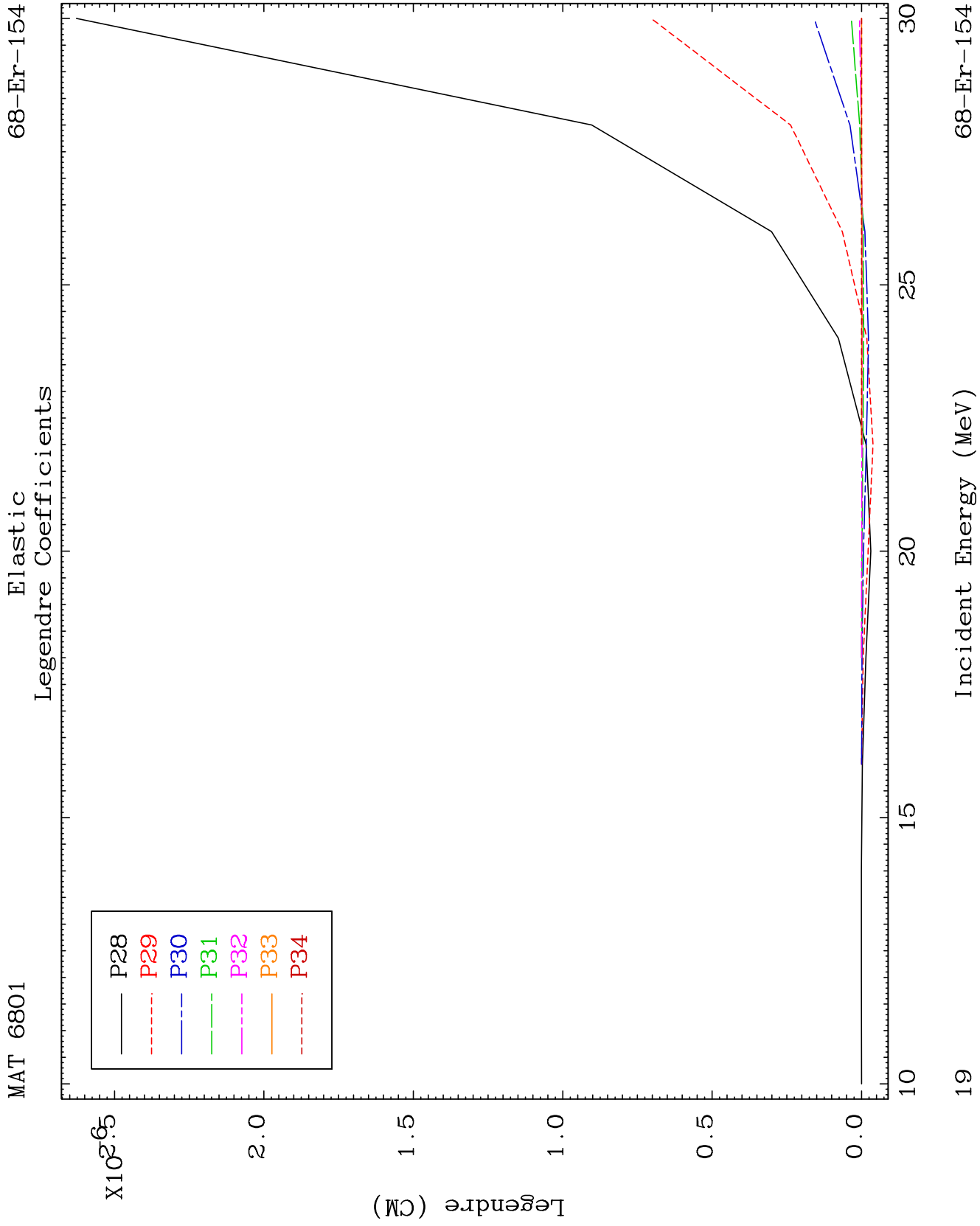
Incident Energy (MeV)

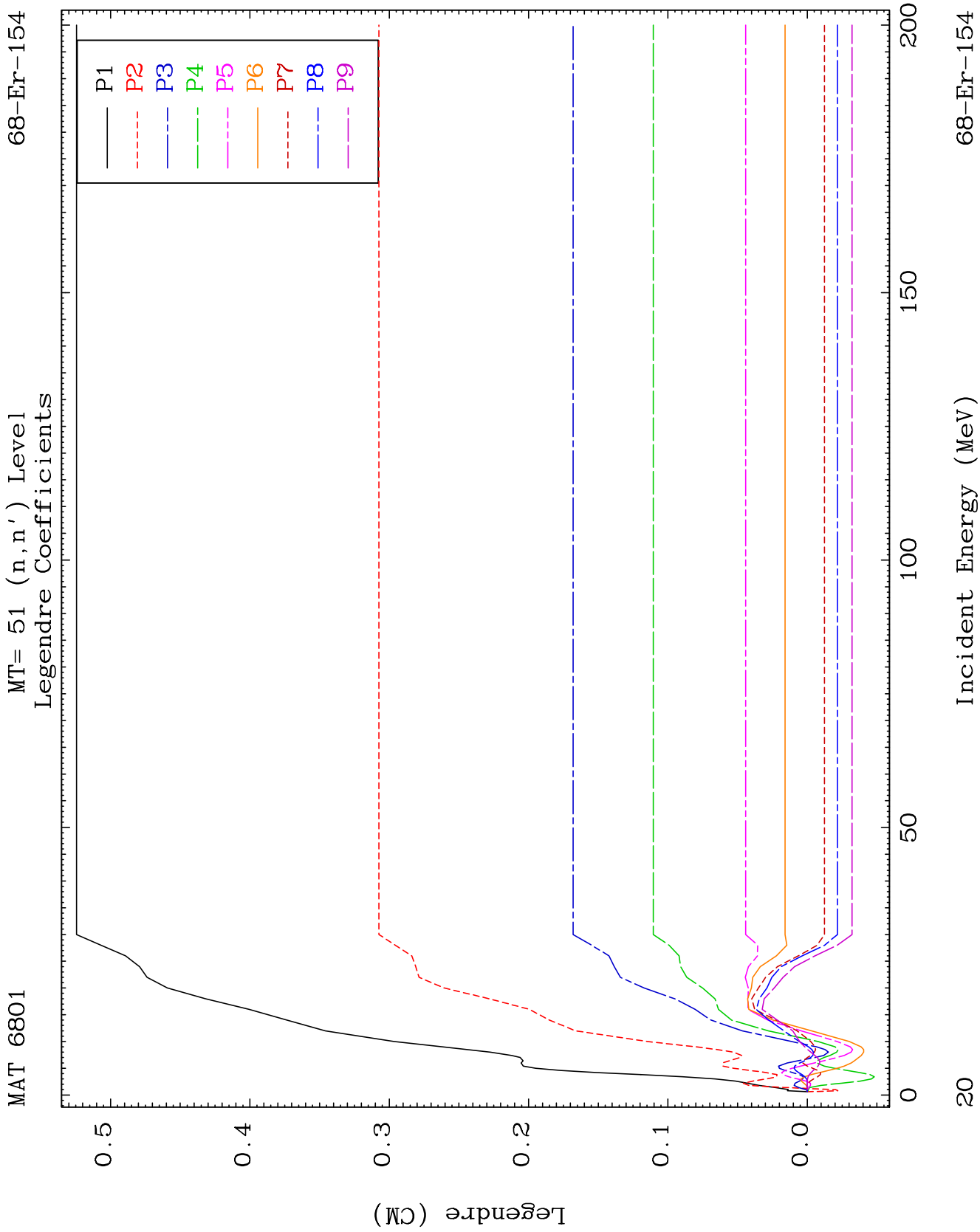
15

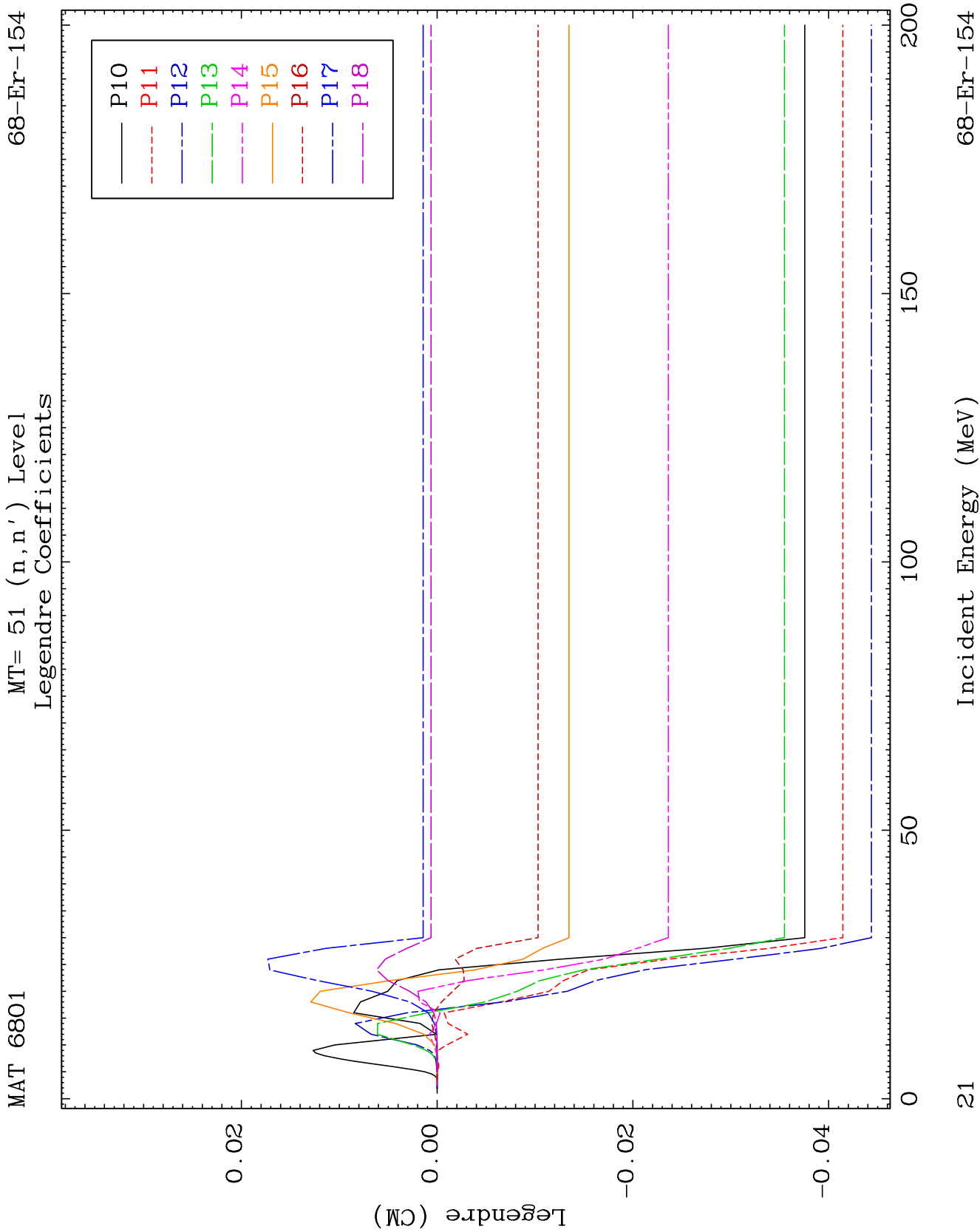


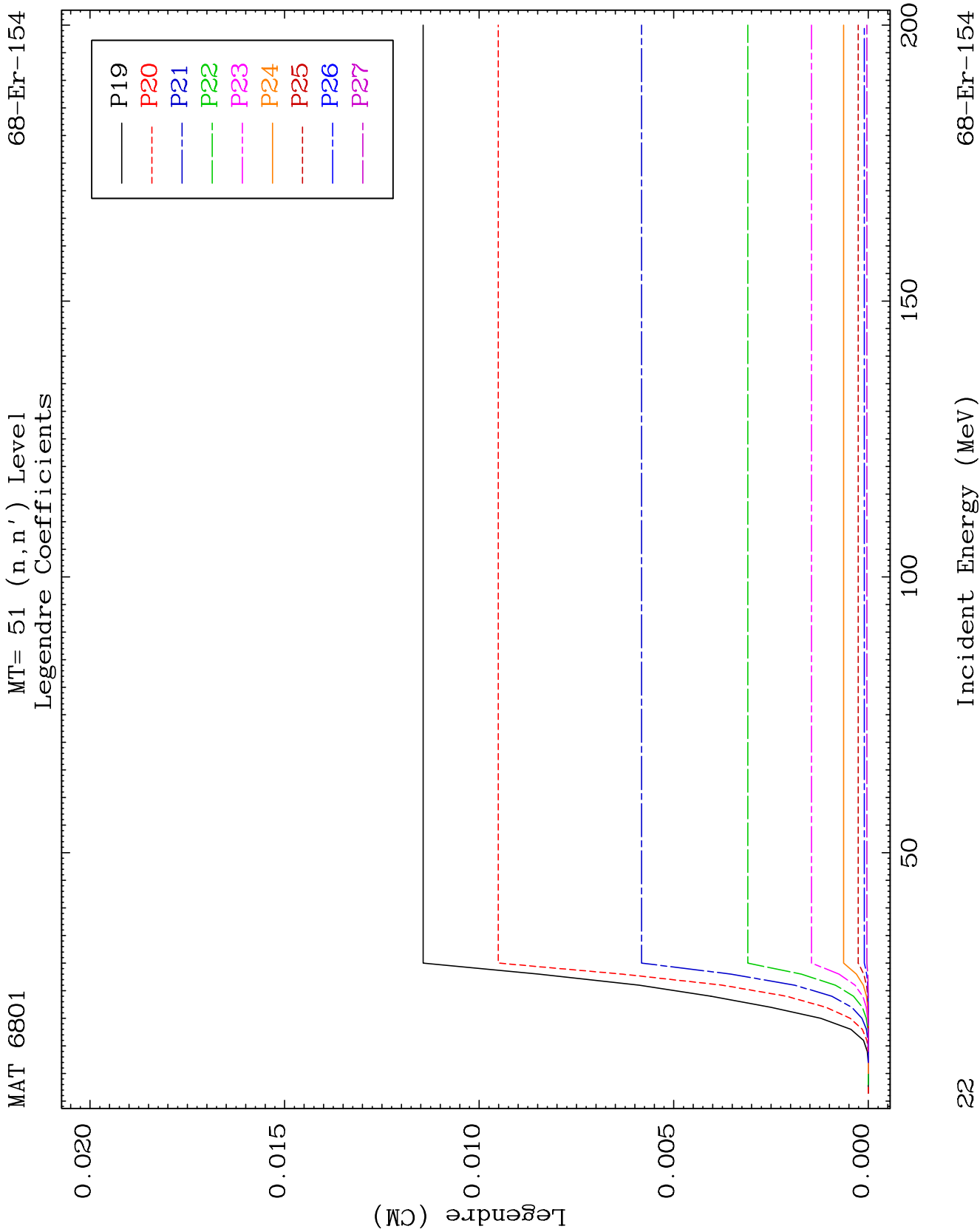


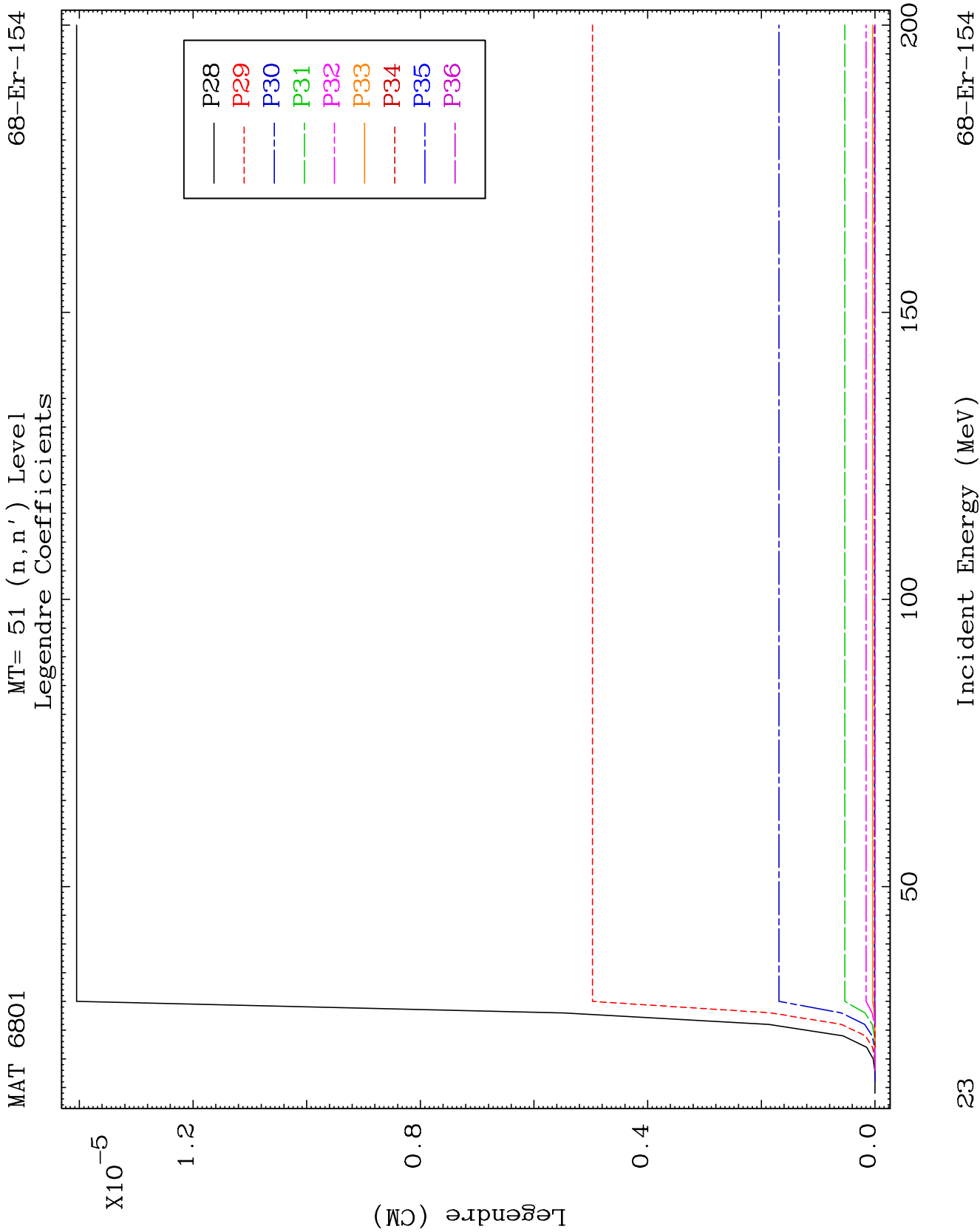




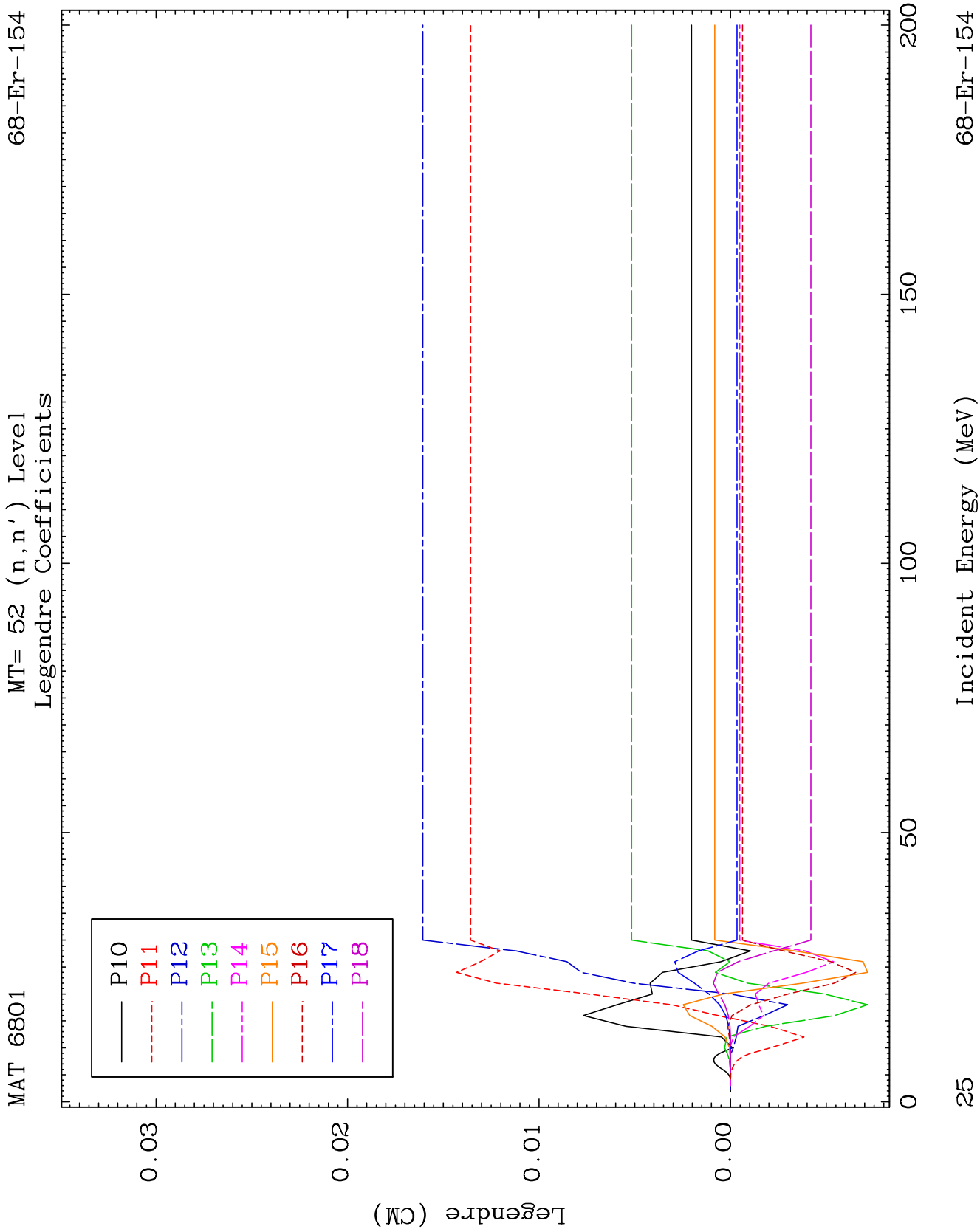


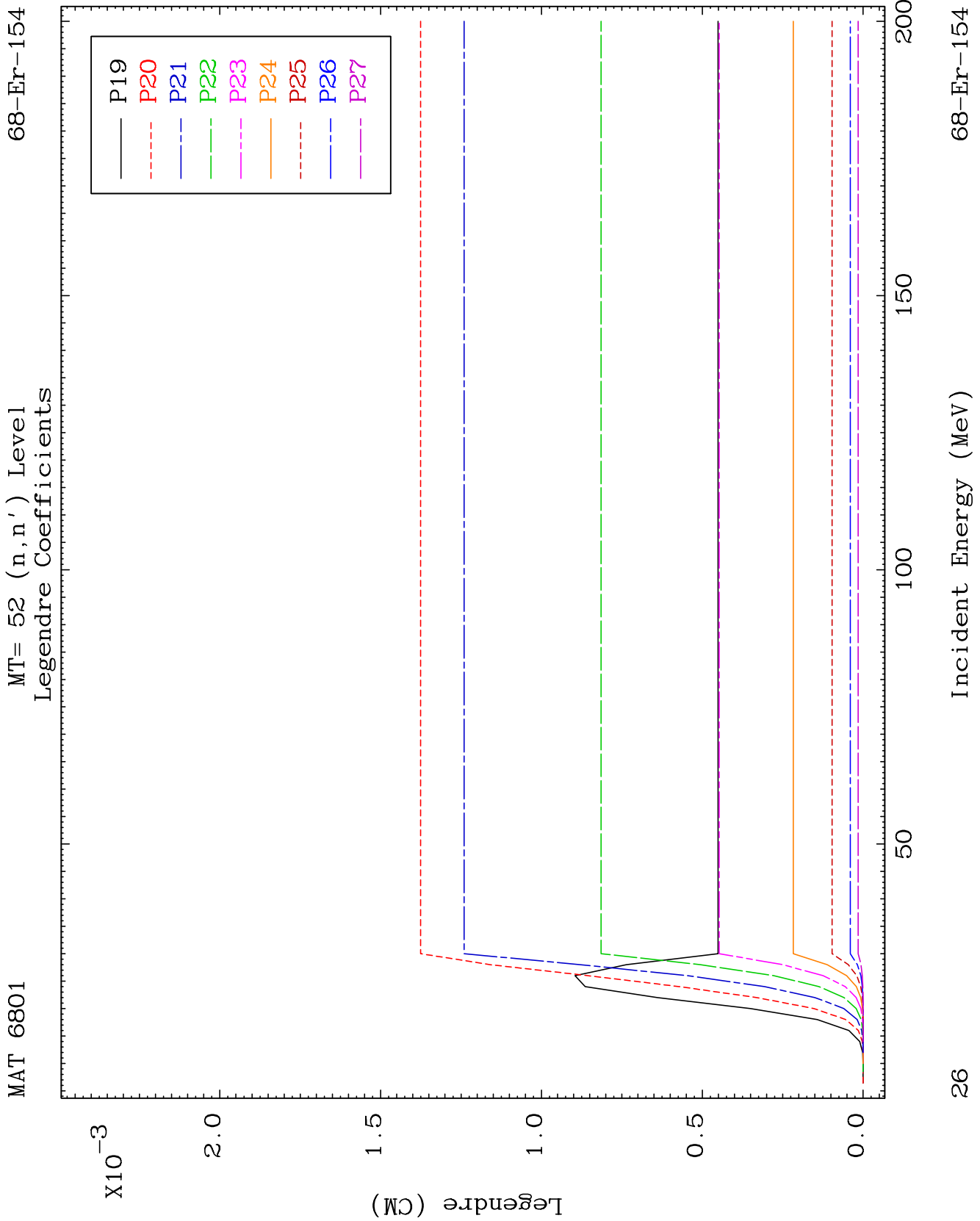


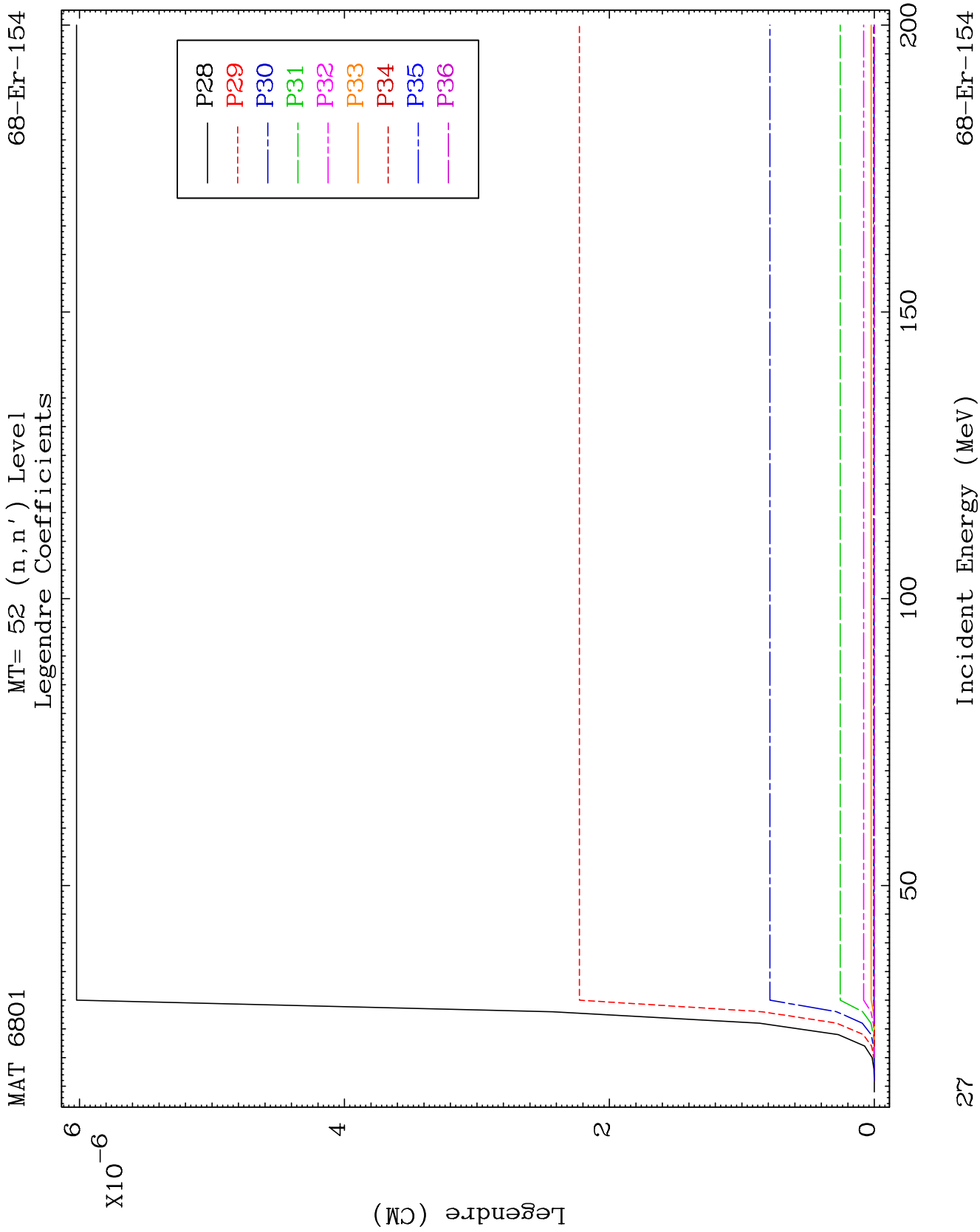










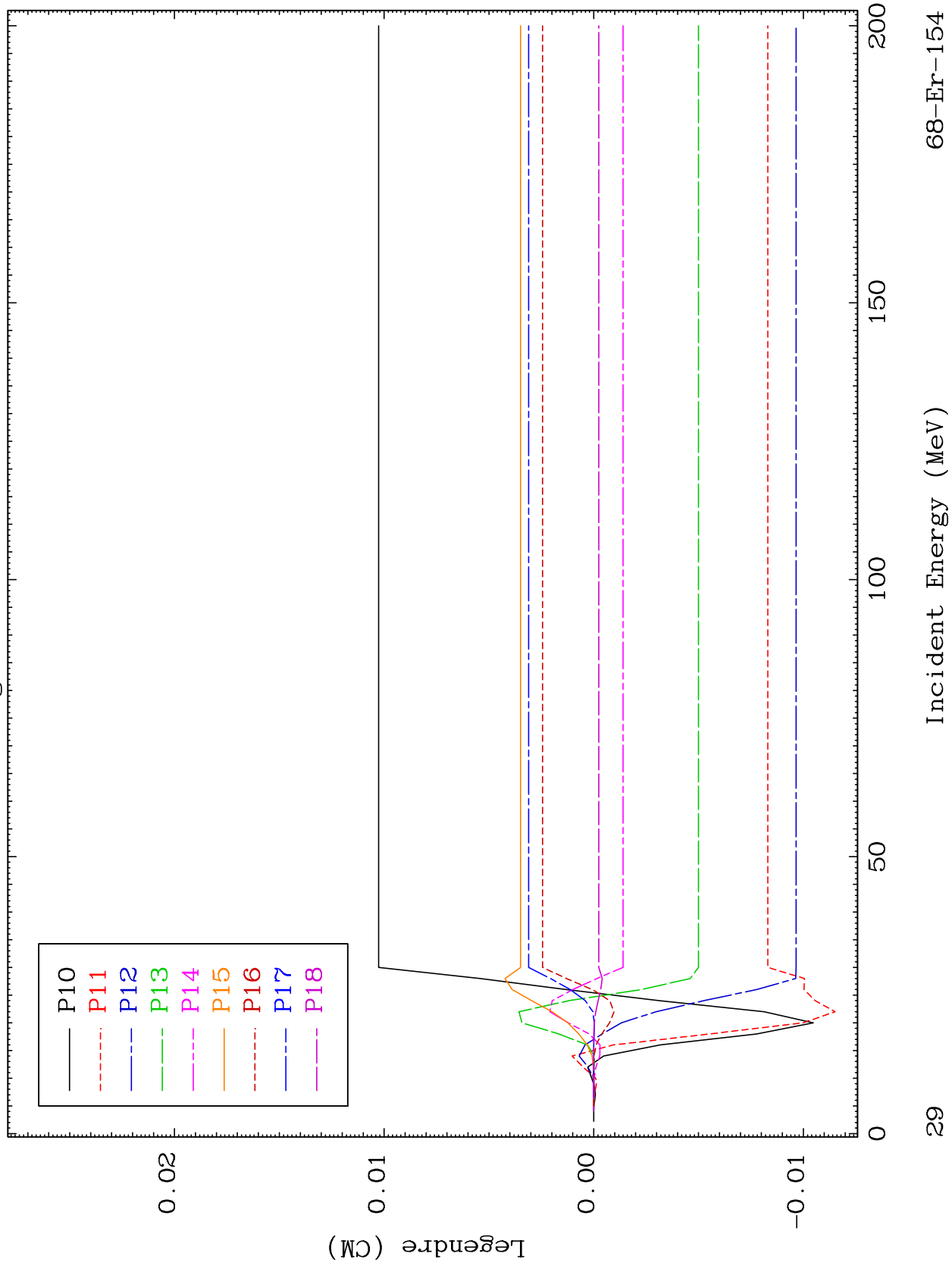


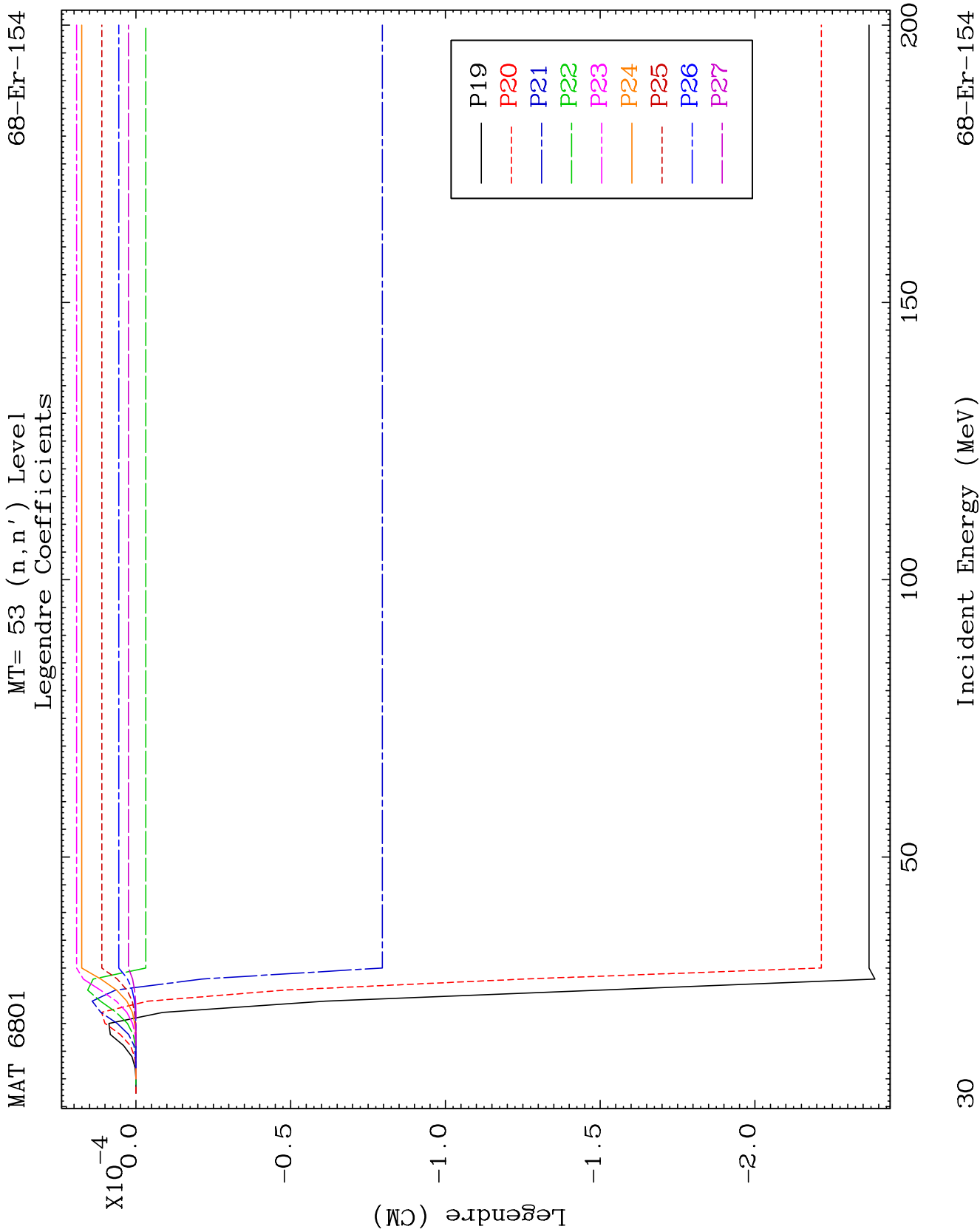


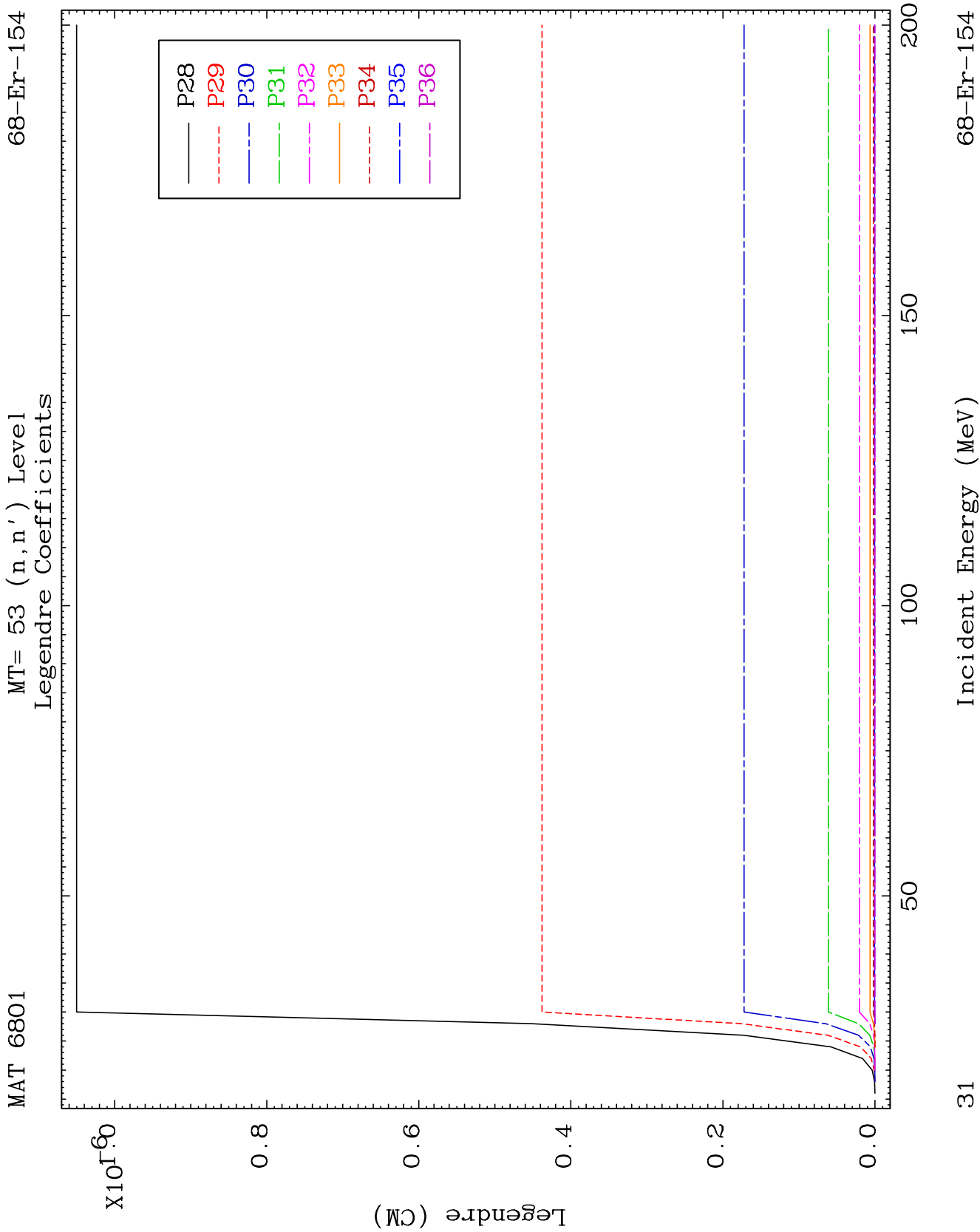
MAT 6801

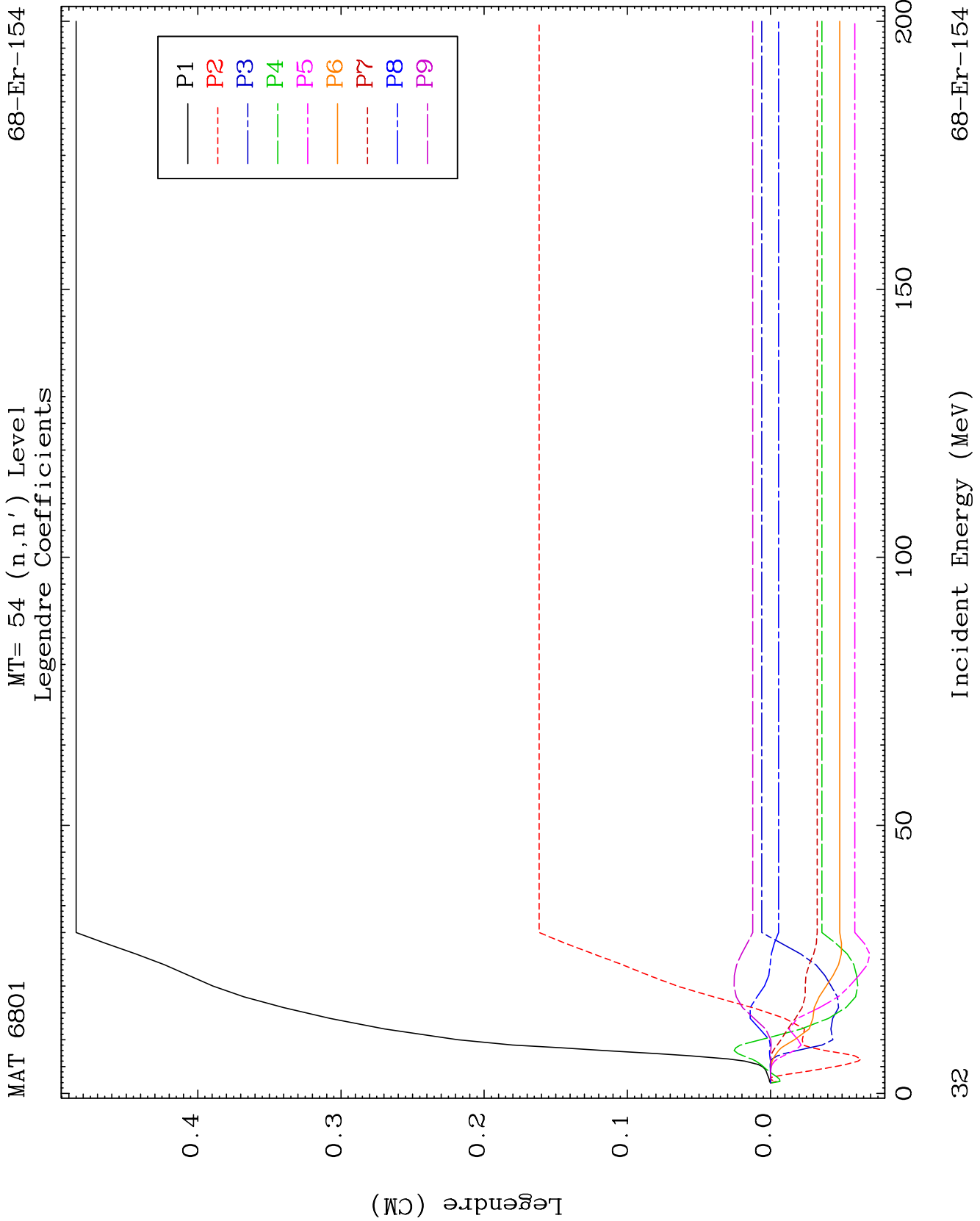
MT= 53 (n, n') Level  
Legendre Coefficients

68-Er-154







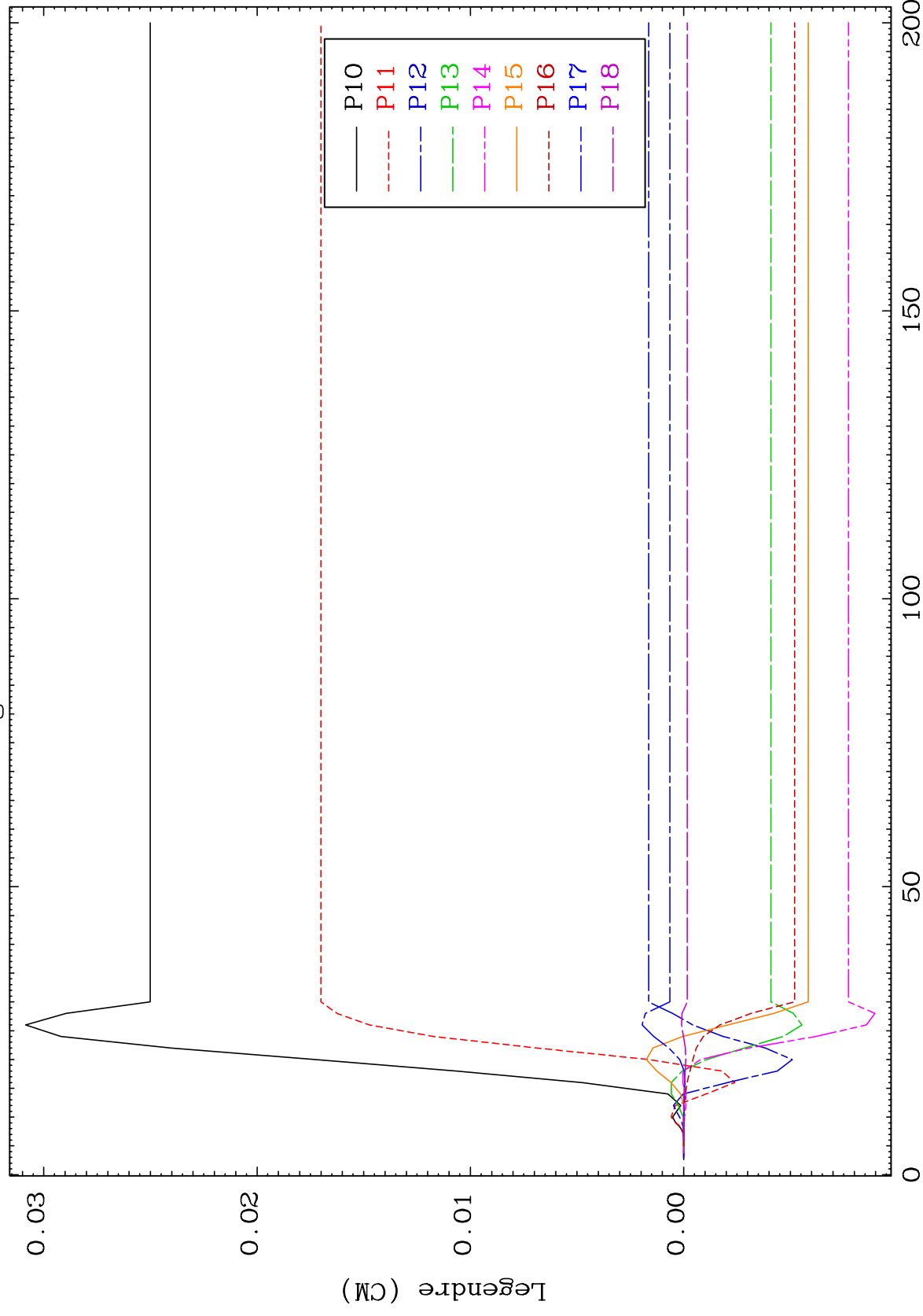


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MT=54 (n, n') Level

68-Er-154

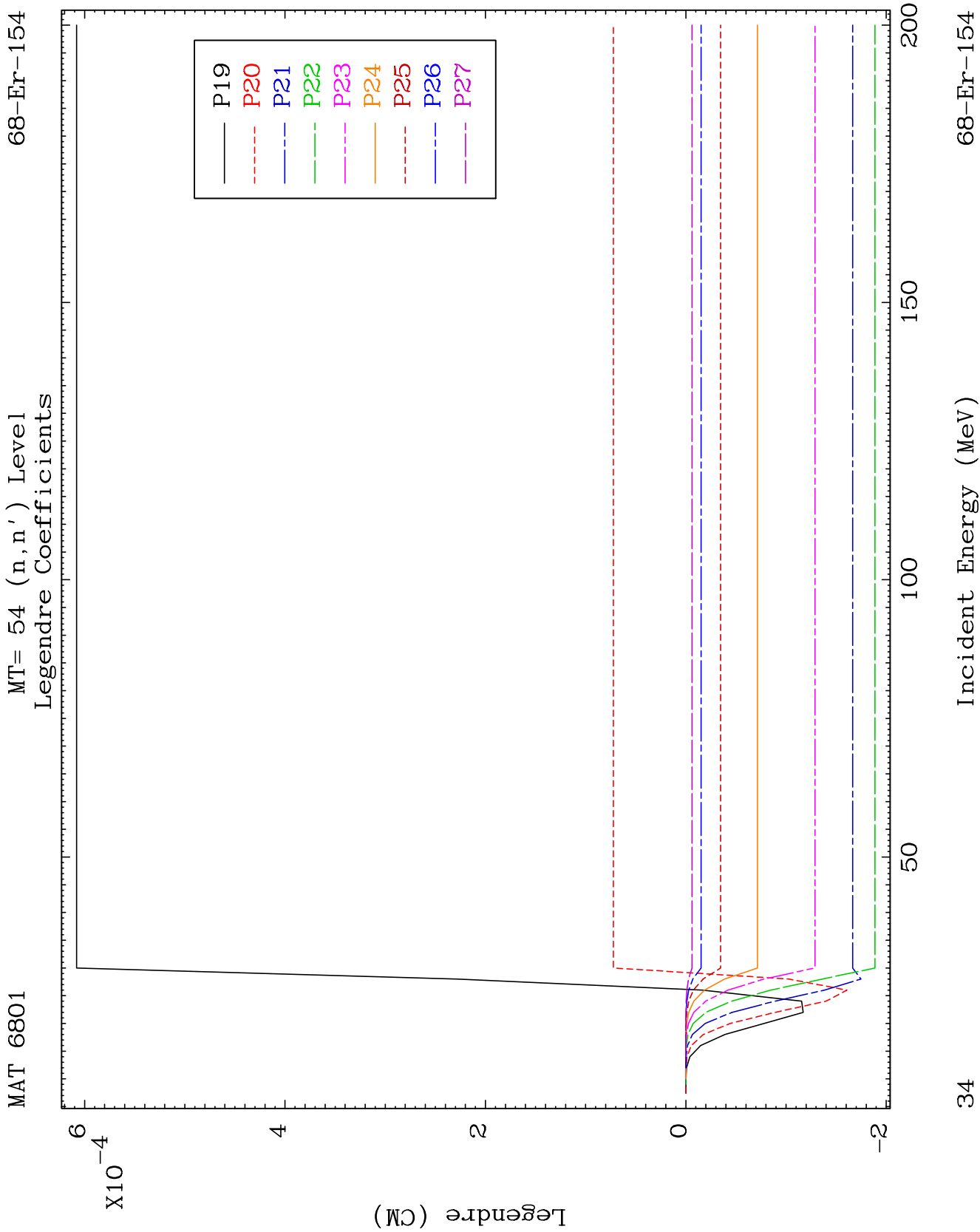
### Legendre Coefficients



33

Incident Energy (MeV)

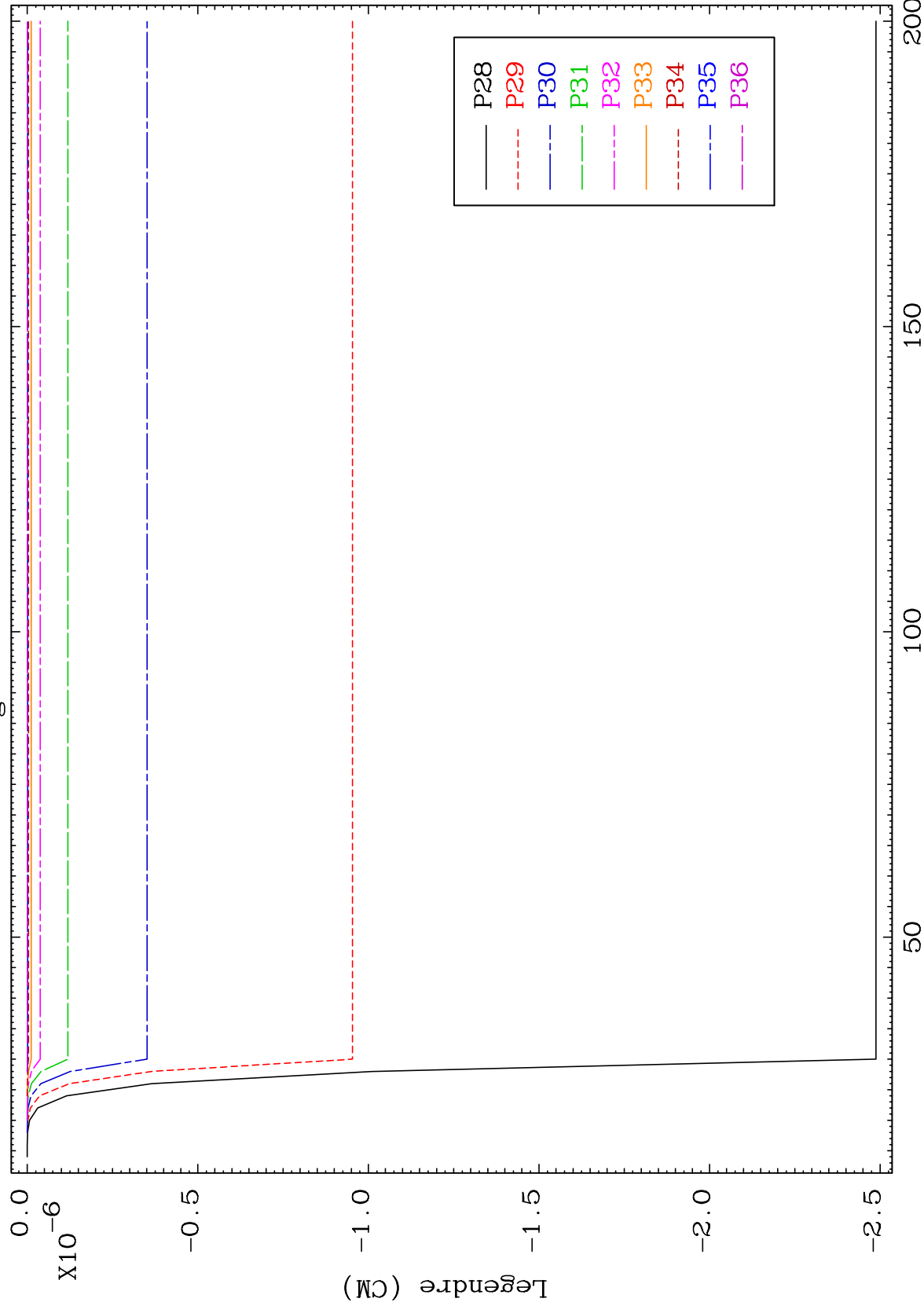
68-Er-154



MAT 6801

MT= 54 (n, n') Level  
Legendre Coefficients

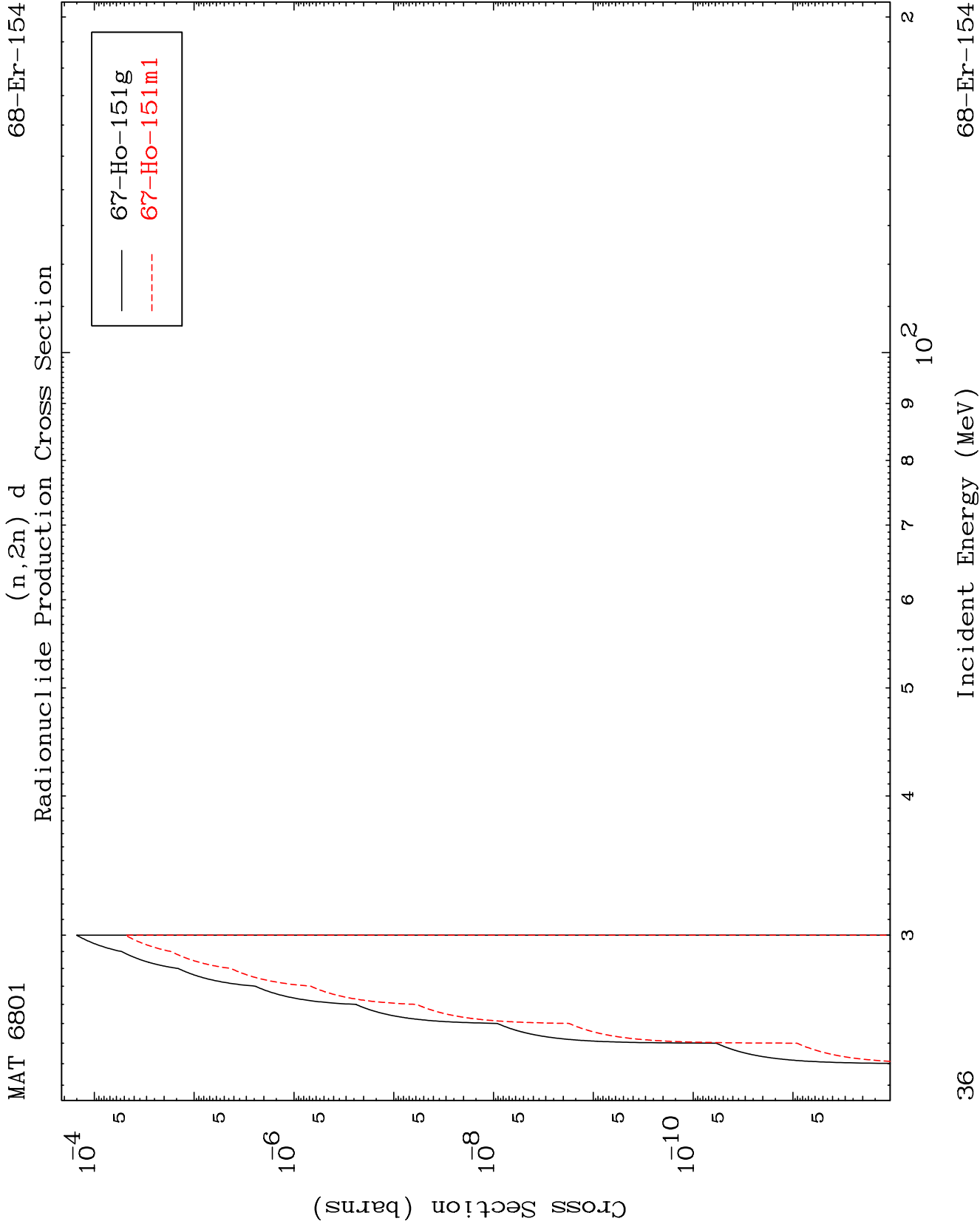
68-Er-154



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Incident Energy (MeV)

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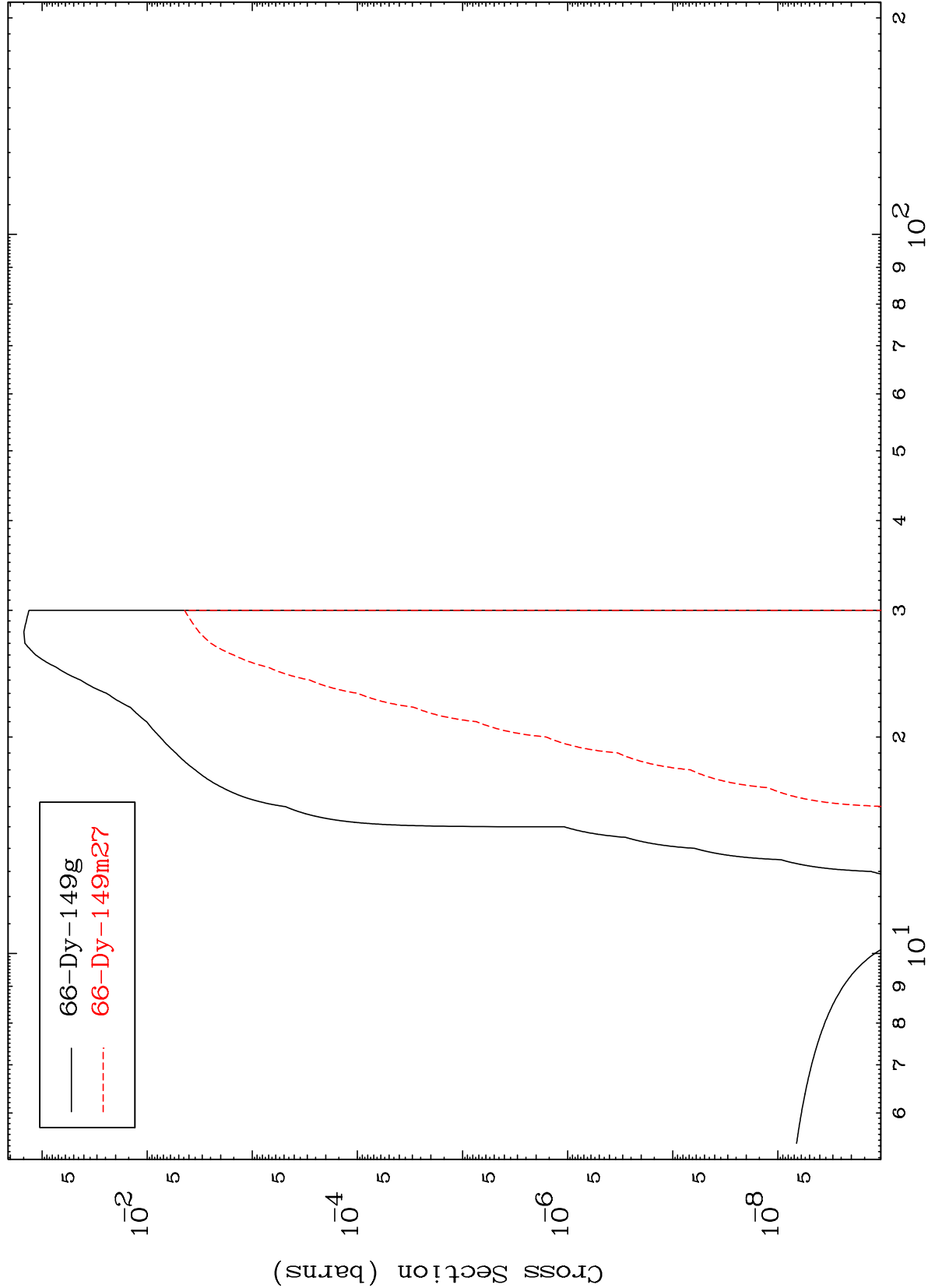


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(n,2n)  $\alpha$

Radionuclide Production Cross Section



37

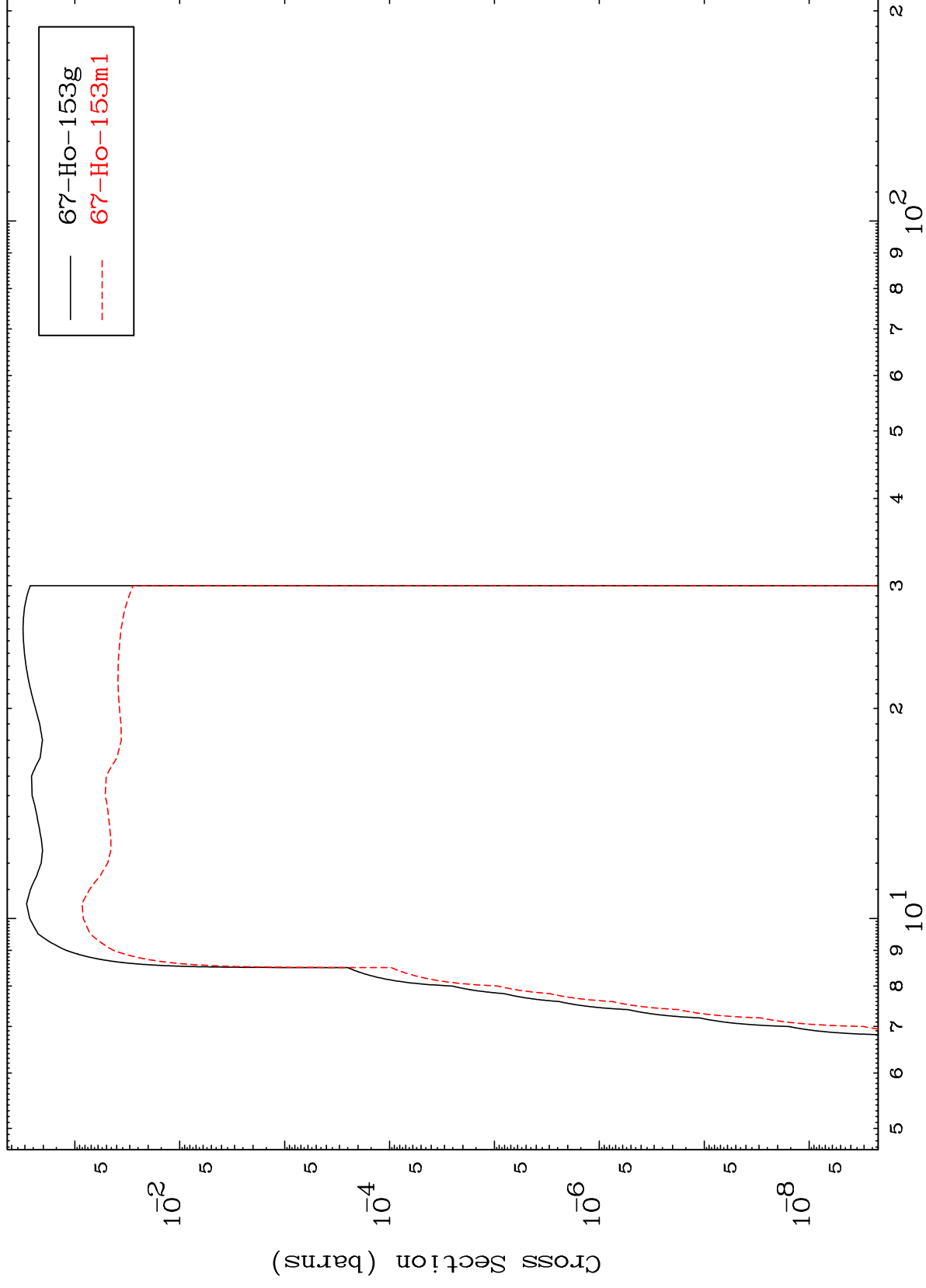
Incident Energy (MeV)

68-Er-154

MAT 6801

68-Er-154

$(n,n')$  p  
Radionuclide Production Cross Section



38

Incident Energy (MeV)

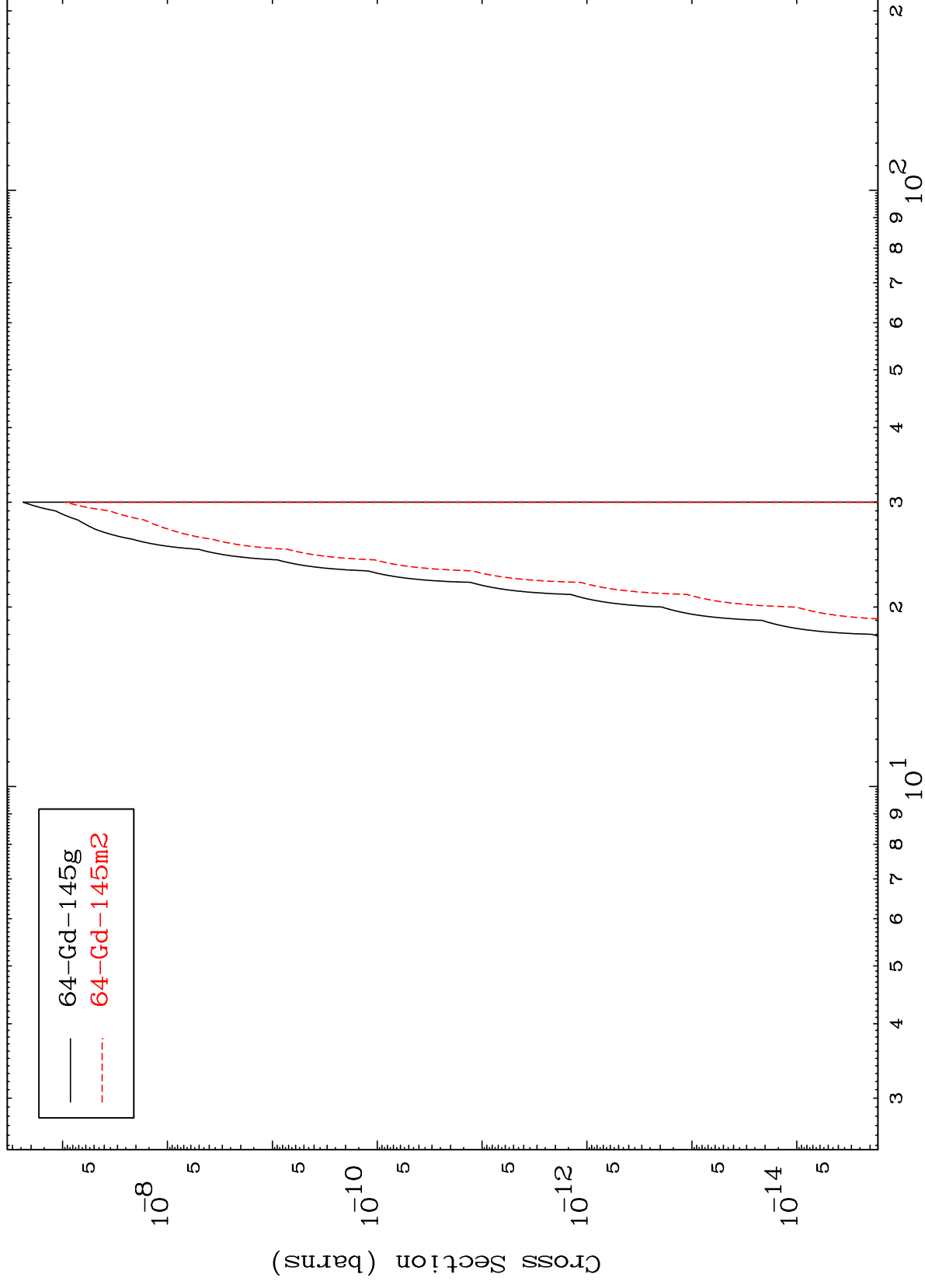
68-Er-154

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(n,2n) 2α

68-Er-154

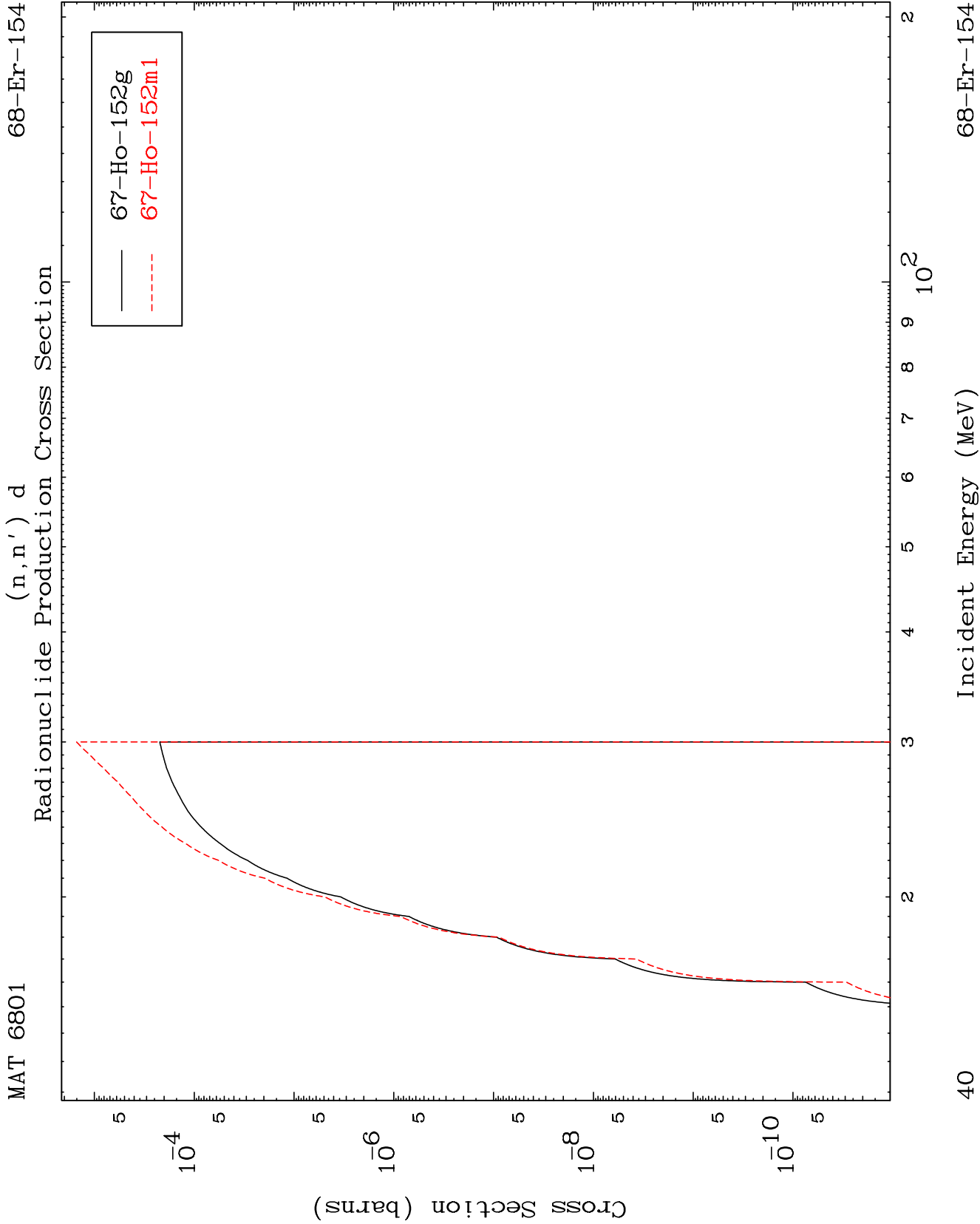
Radionuclide Production Cross Section



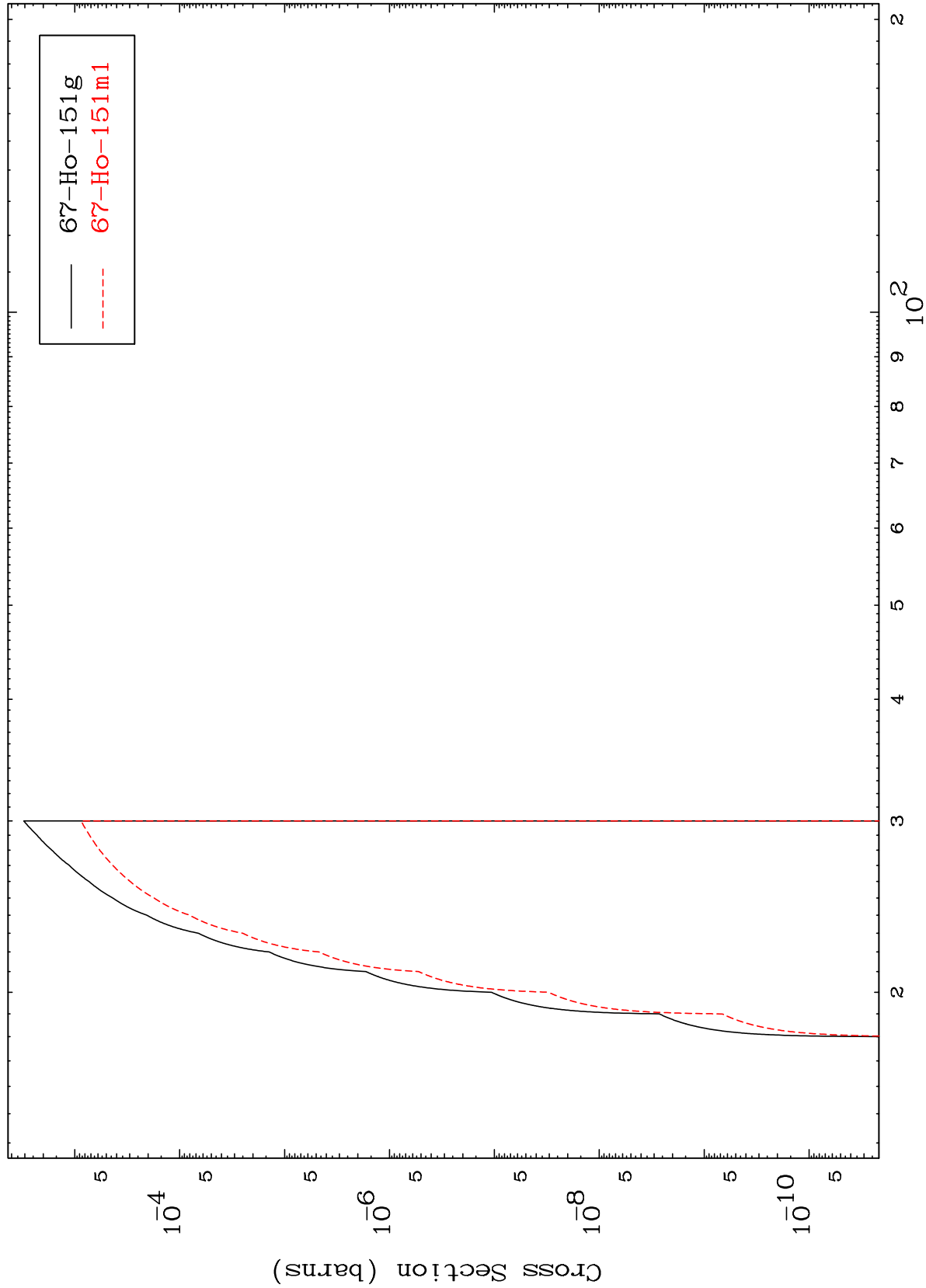
39

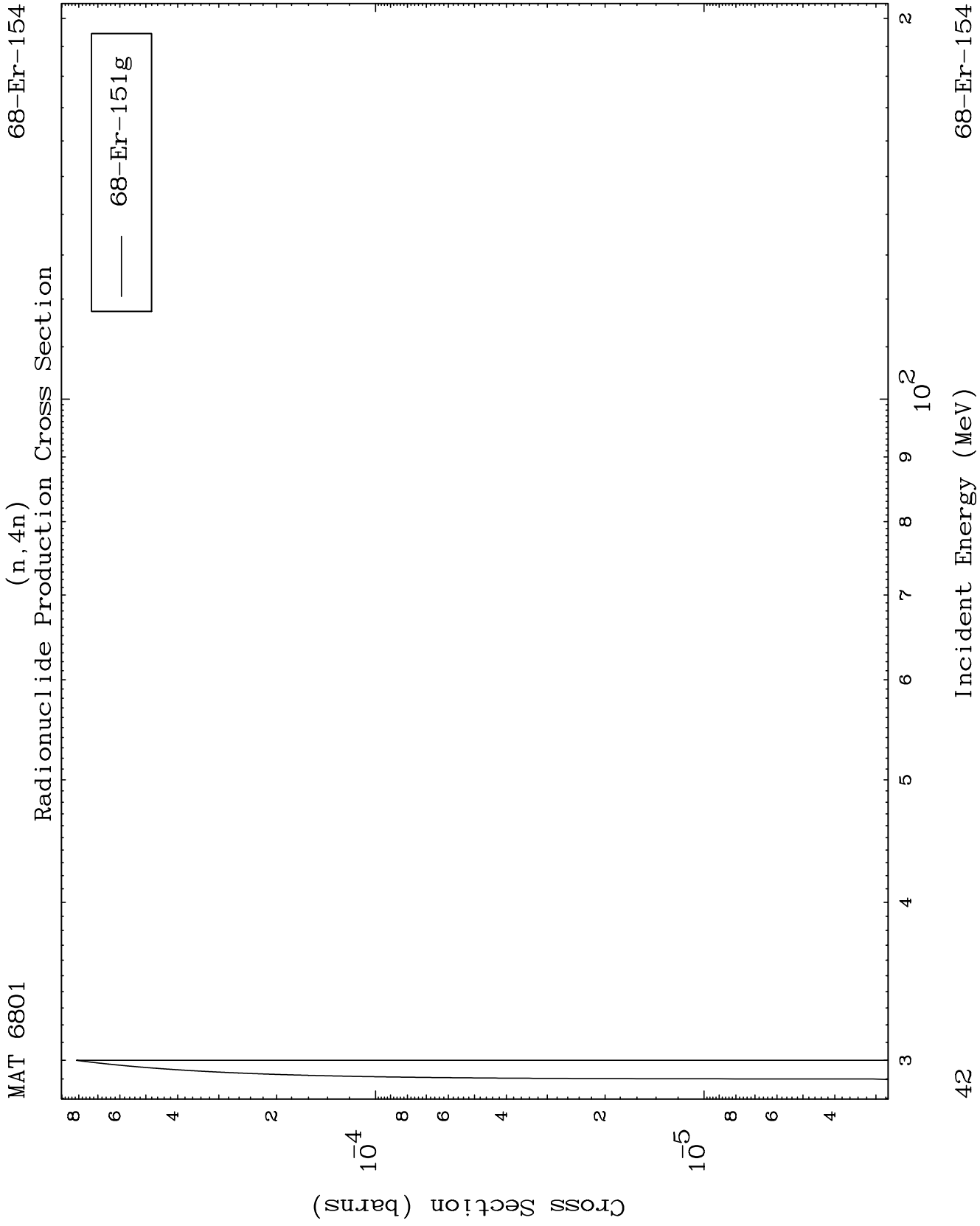
Incident Energy (MeV)

68-Er-154



Radionuclide Production Cross Section

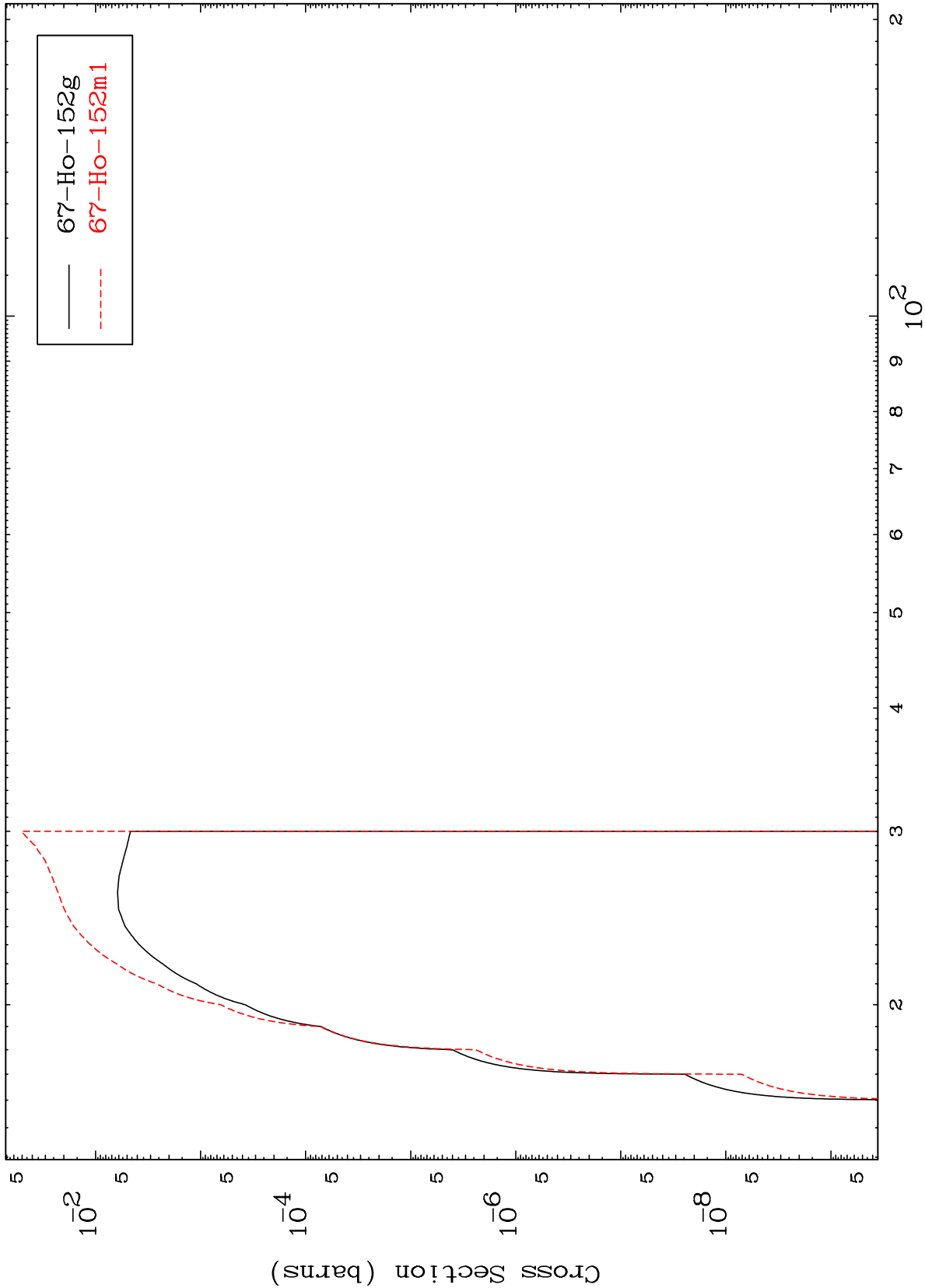




MAT 6801

68-Er-154

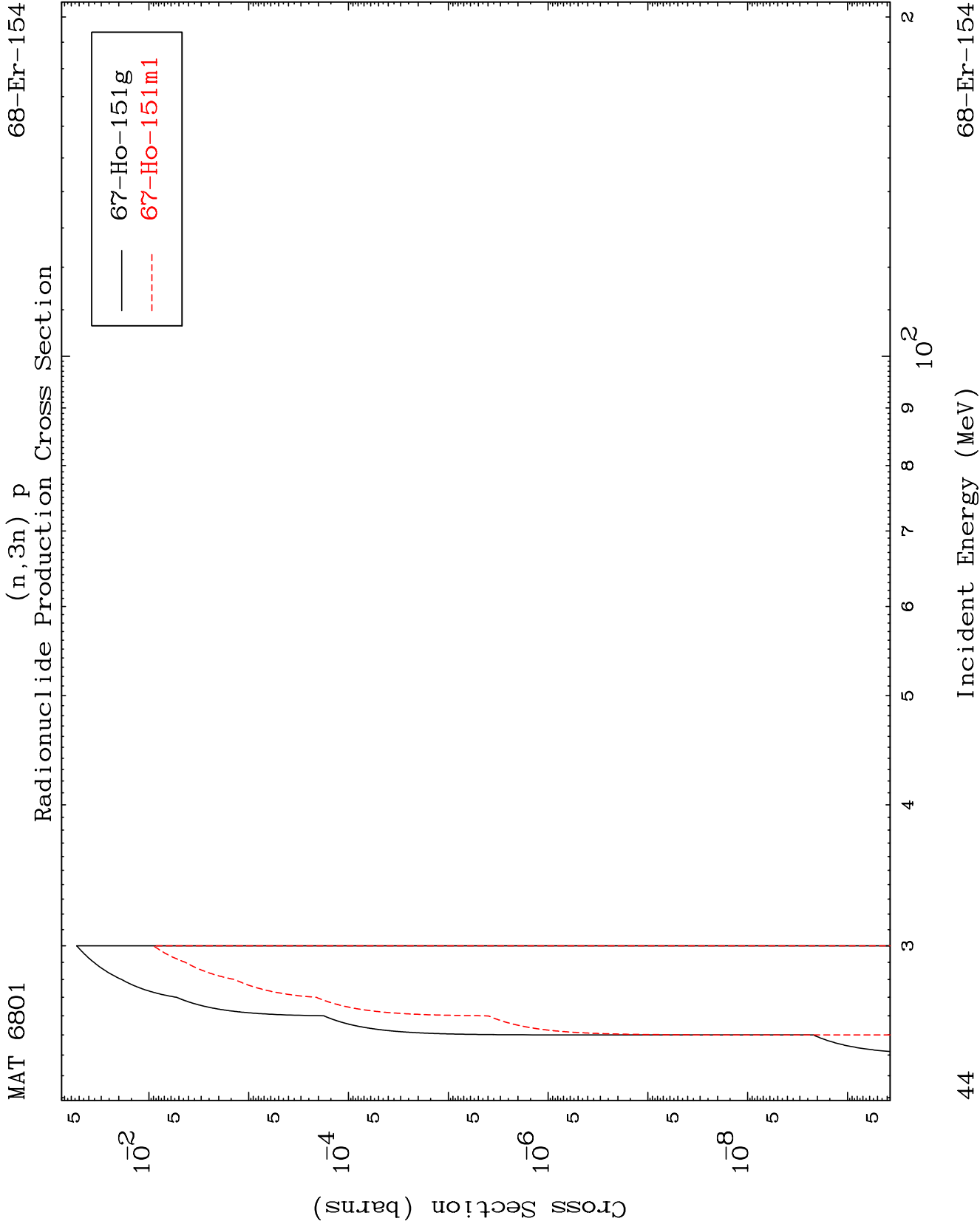
(n,2n) p  
Radionuclide Production Cross Section



43

Incident Energy (MeV)

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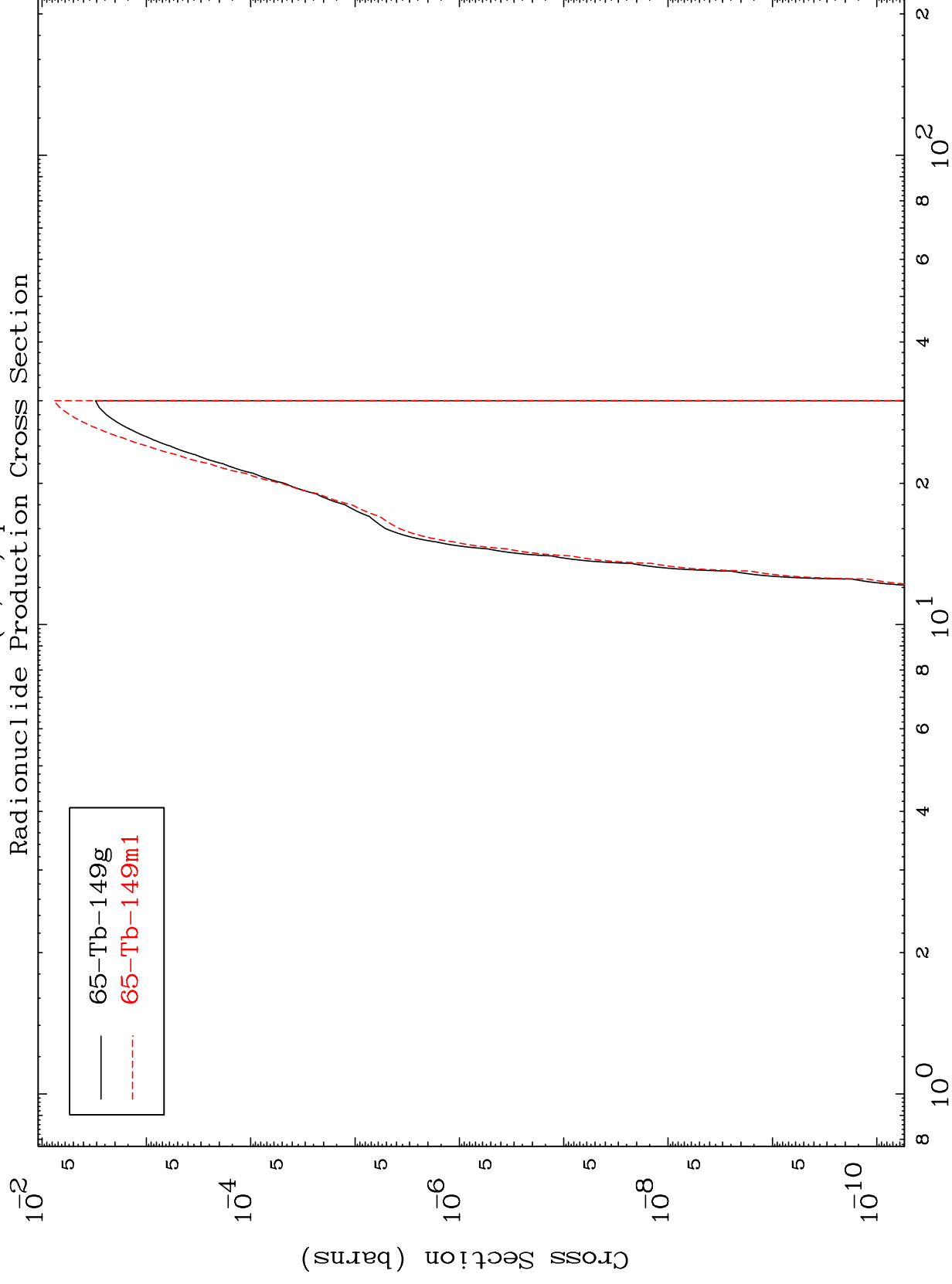


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$$(n, n') \propto p$$

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Radionuclide Production Cross Section



45

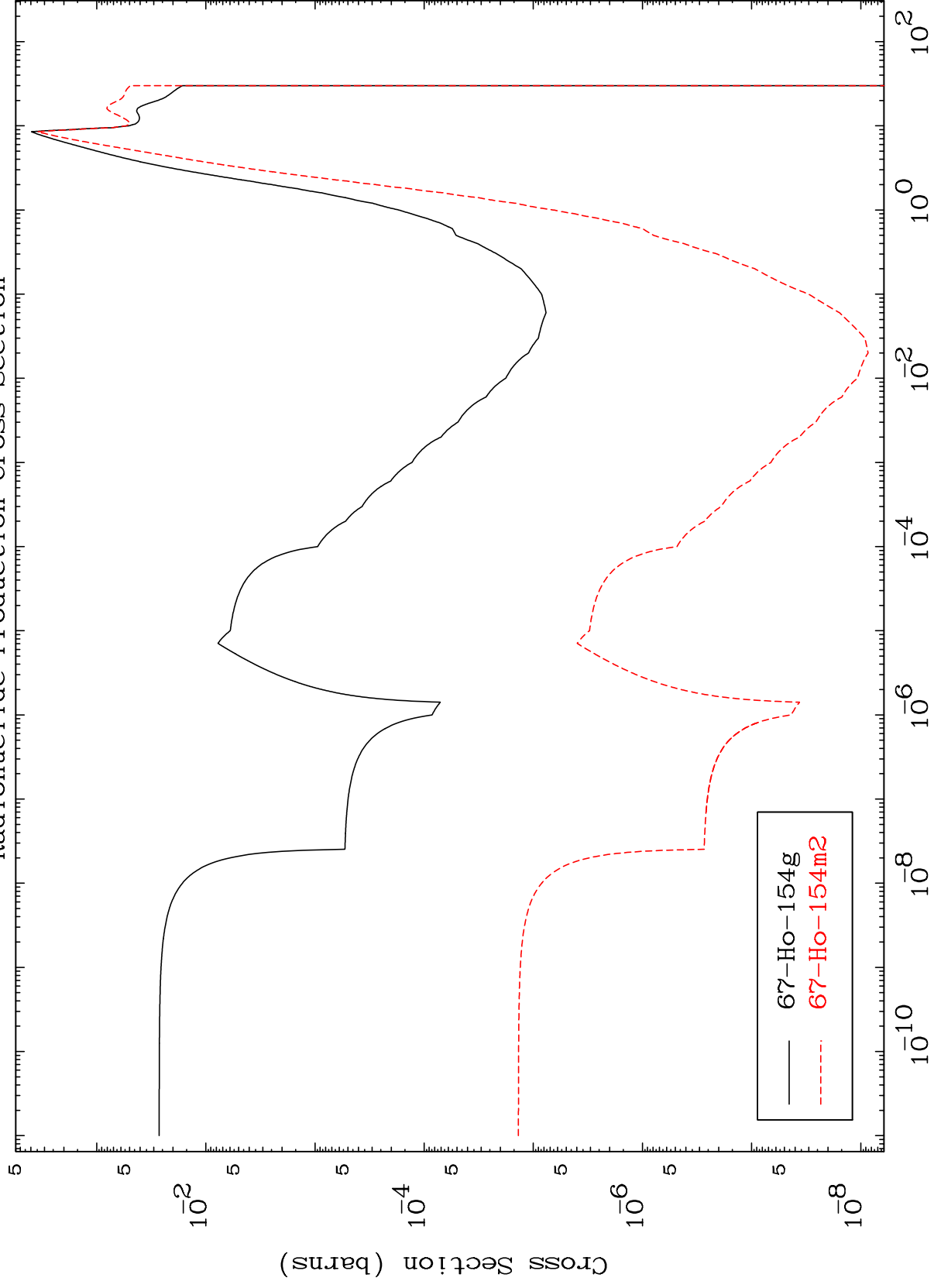
Incident Energy (MeV)

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(n,p)  
Radionuclide Production Cross Section



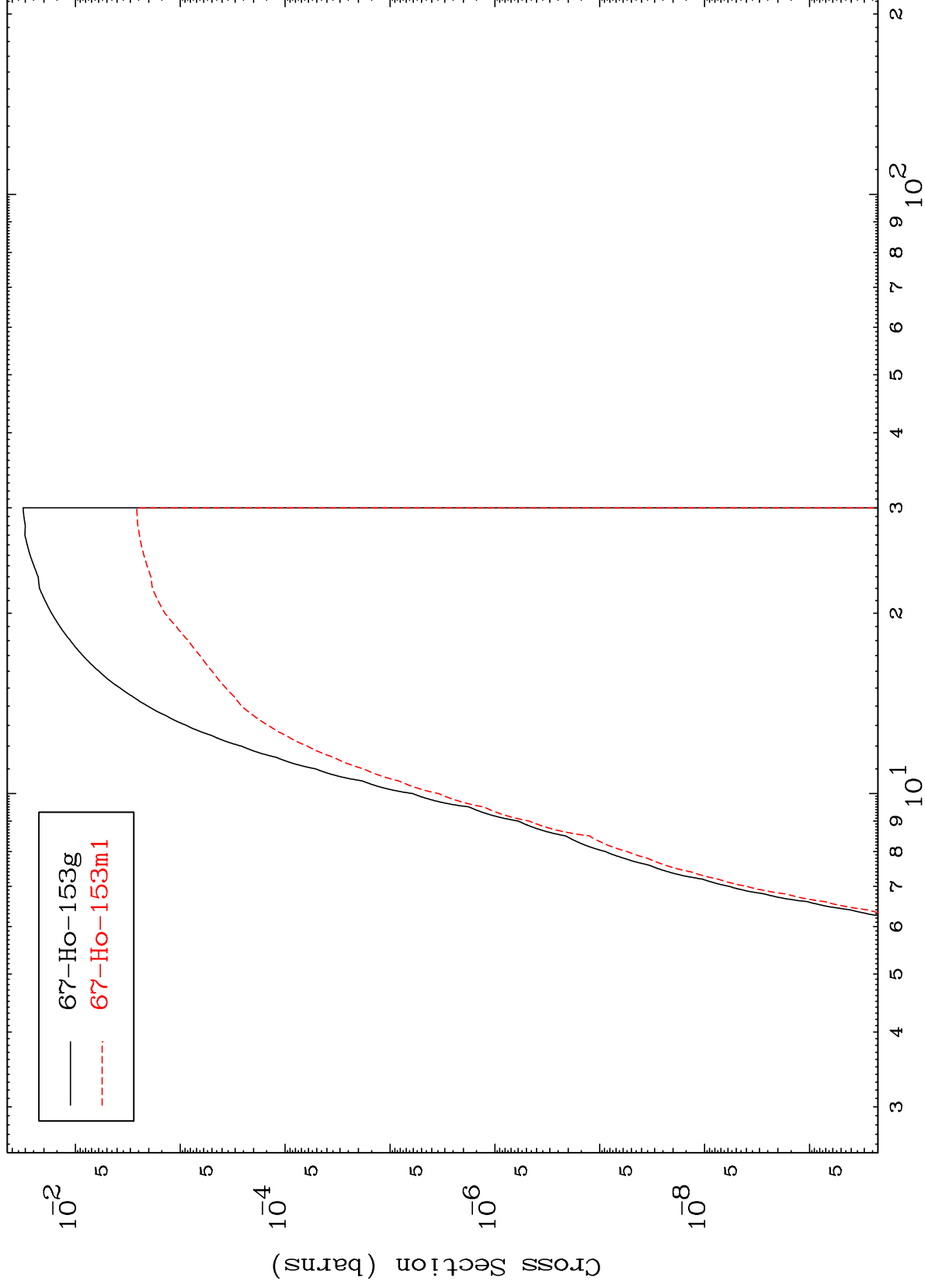
68-Er-154

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68-Er-154

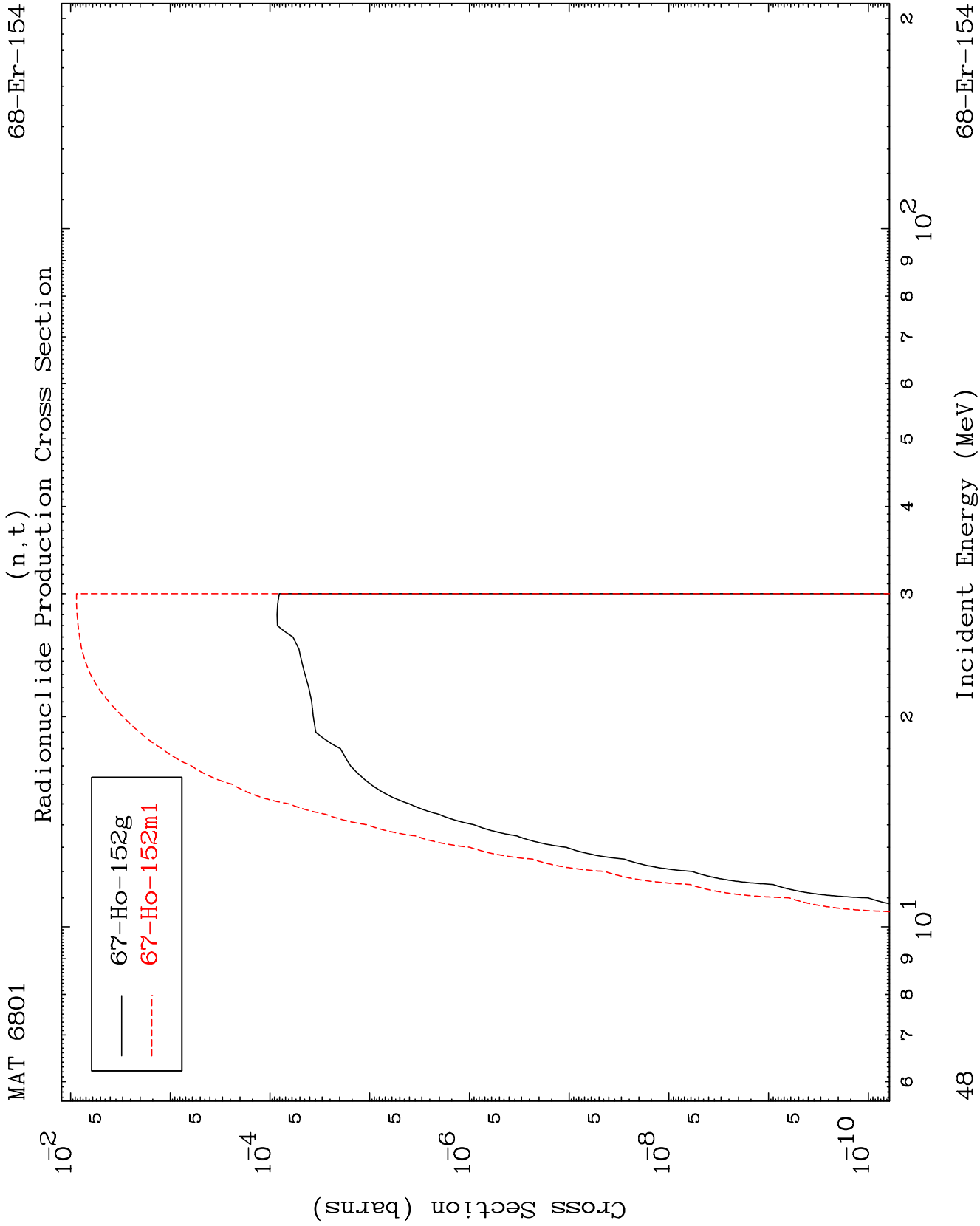
Radionuclide Production Cross Section  
(n,d)



47

Incident Energy (MeV)

68-Er-154

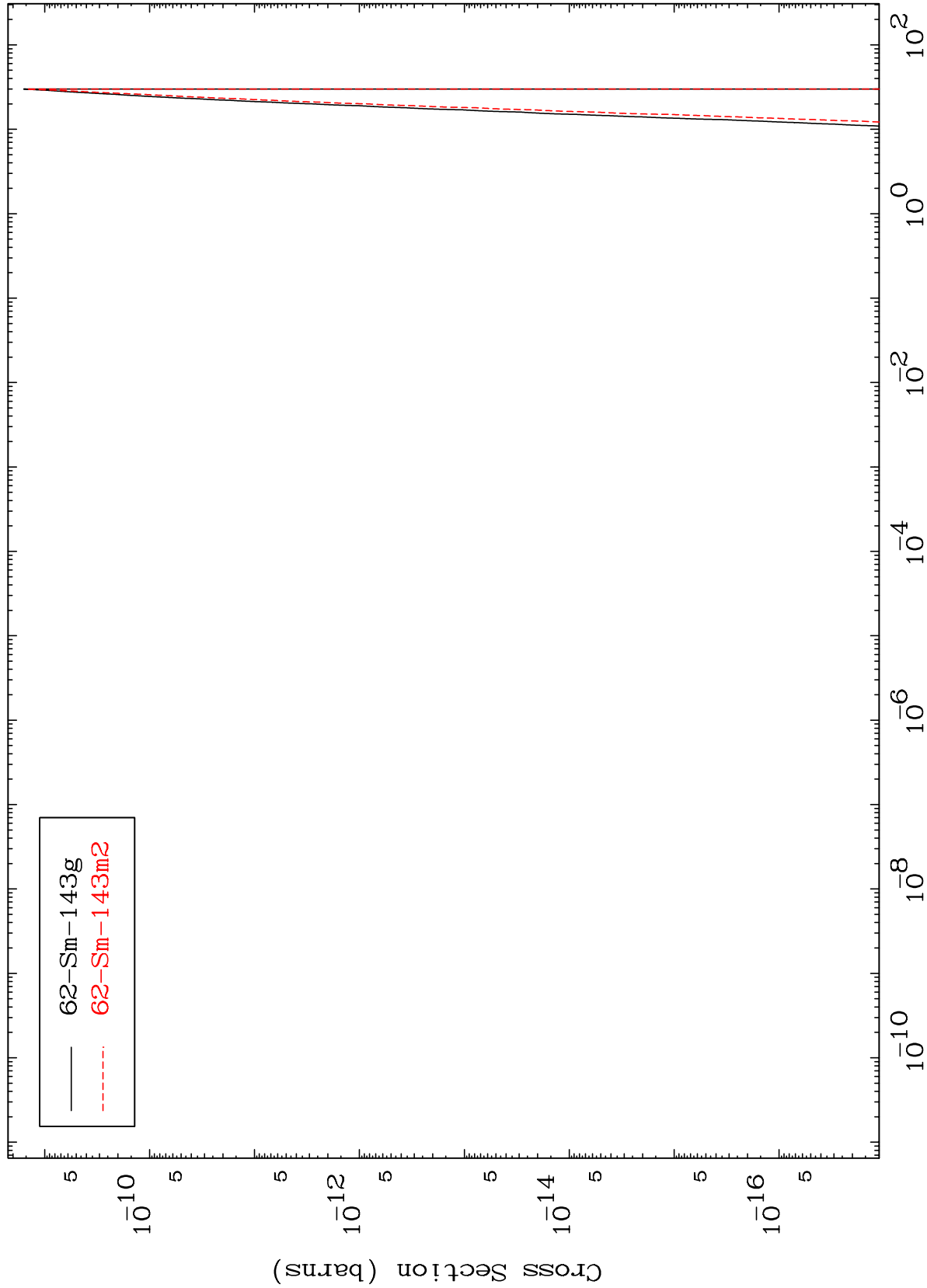
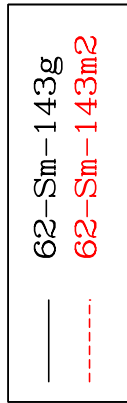


MAT 6801

(n,3α)

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Radionuclide Production Cross Section



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