

Program EVALPLOT
(Version 2021-1)

by

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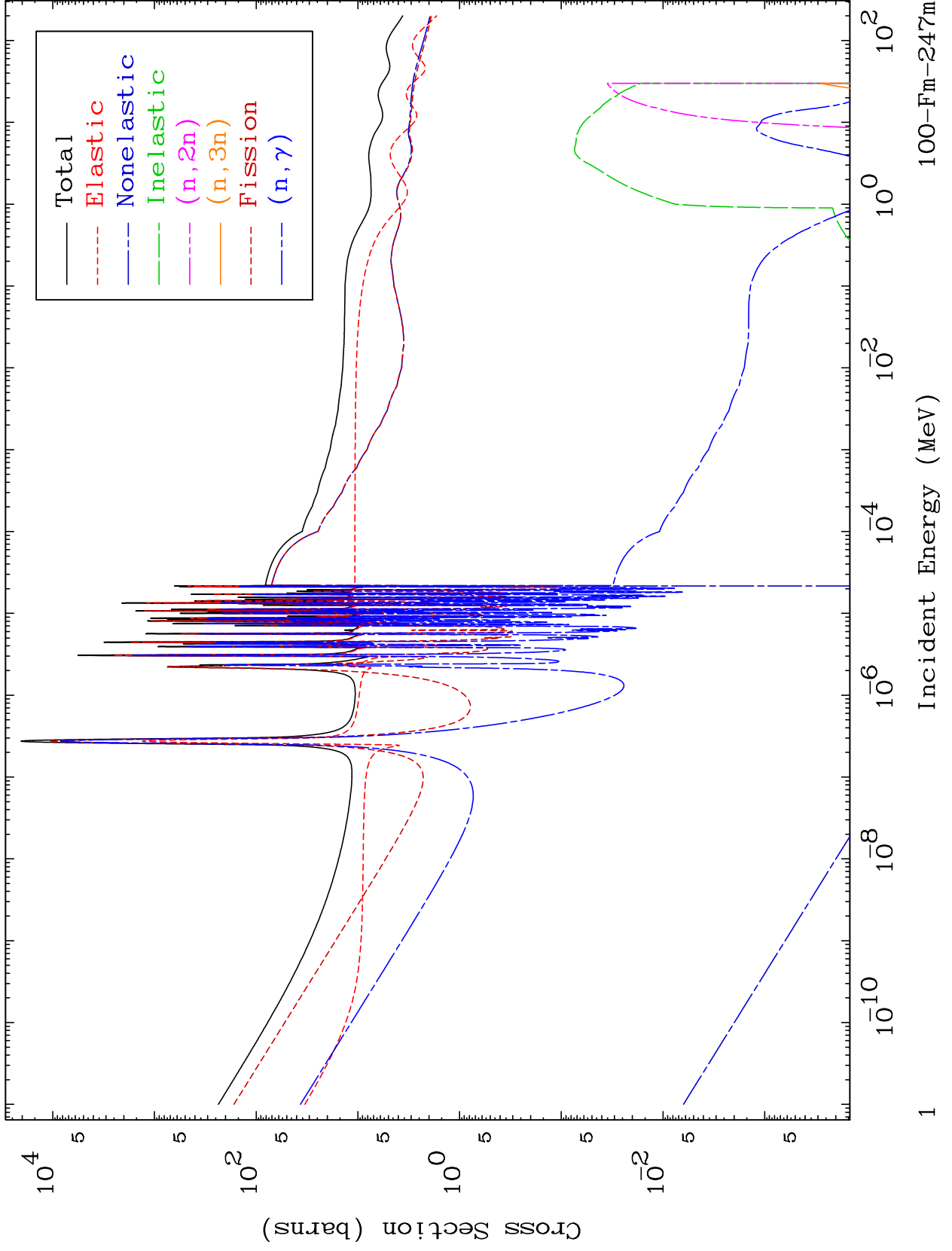
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9933

Neutron Major
293 Kelvin Cross Sections

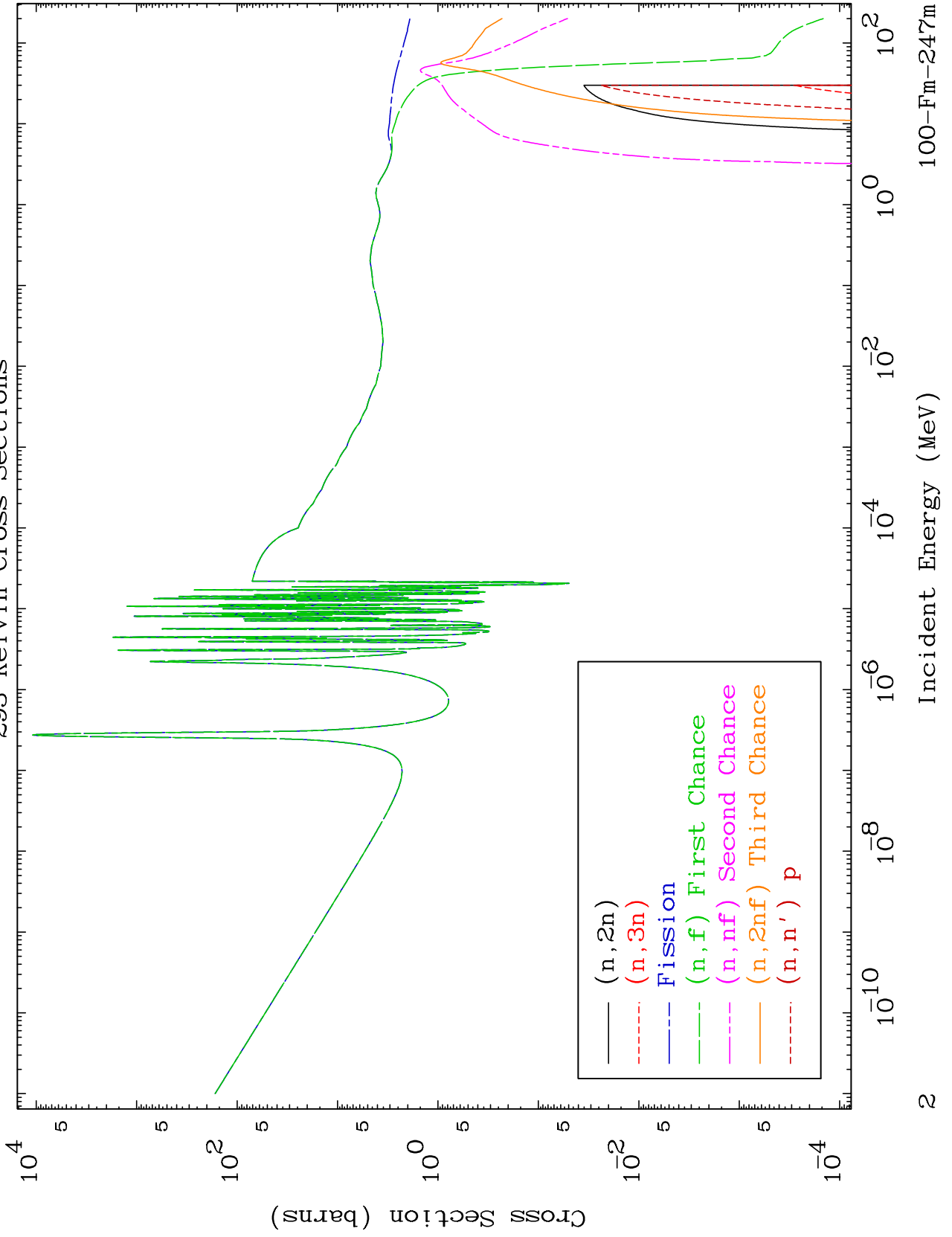
100-Fm-247m



MAT 9933

Neutron Absorption
293 Kelvin Cross Sections

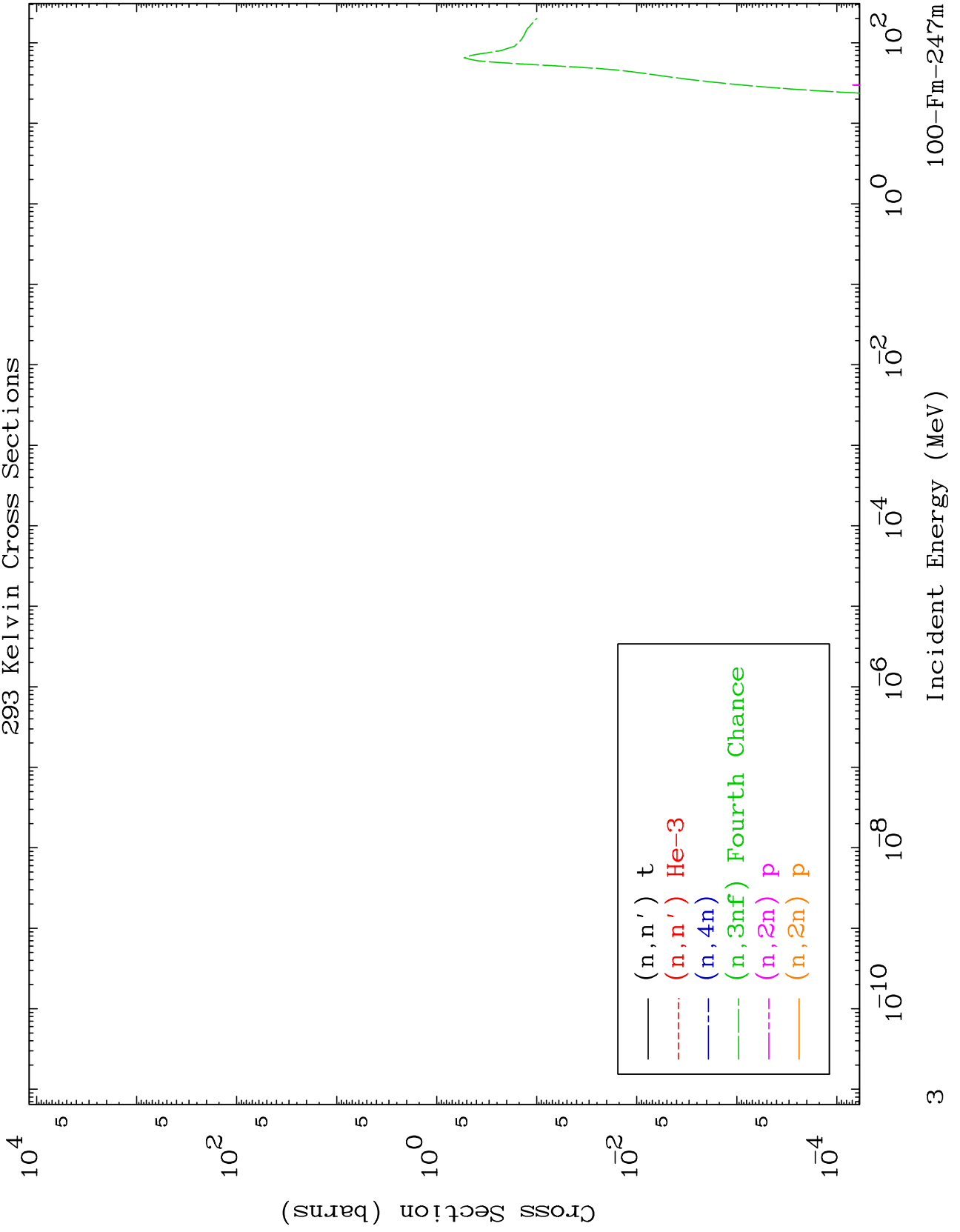
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MAT 9933

Neutron Absorption
293 Kelvin Cross Sections

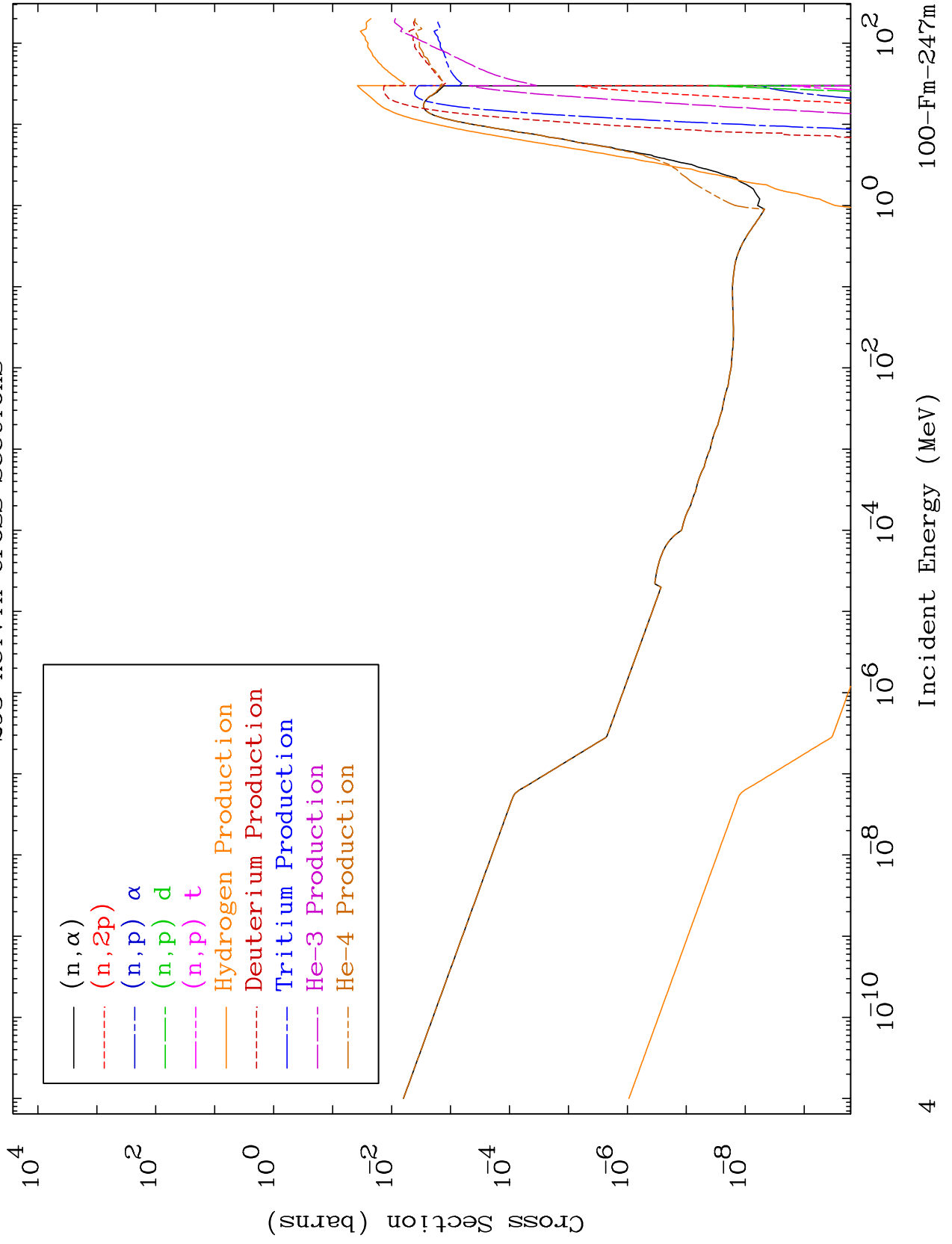
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Neutron Absorption
293 Kelvin Cross Sections

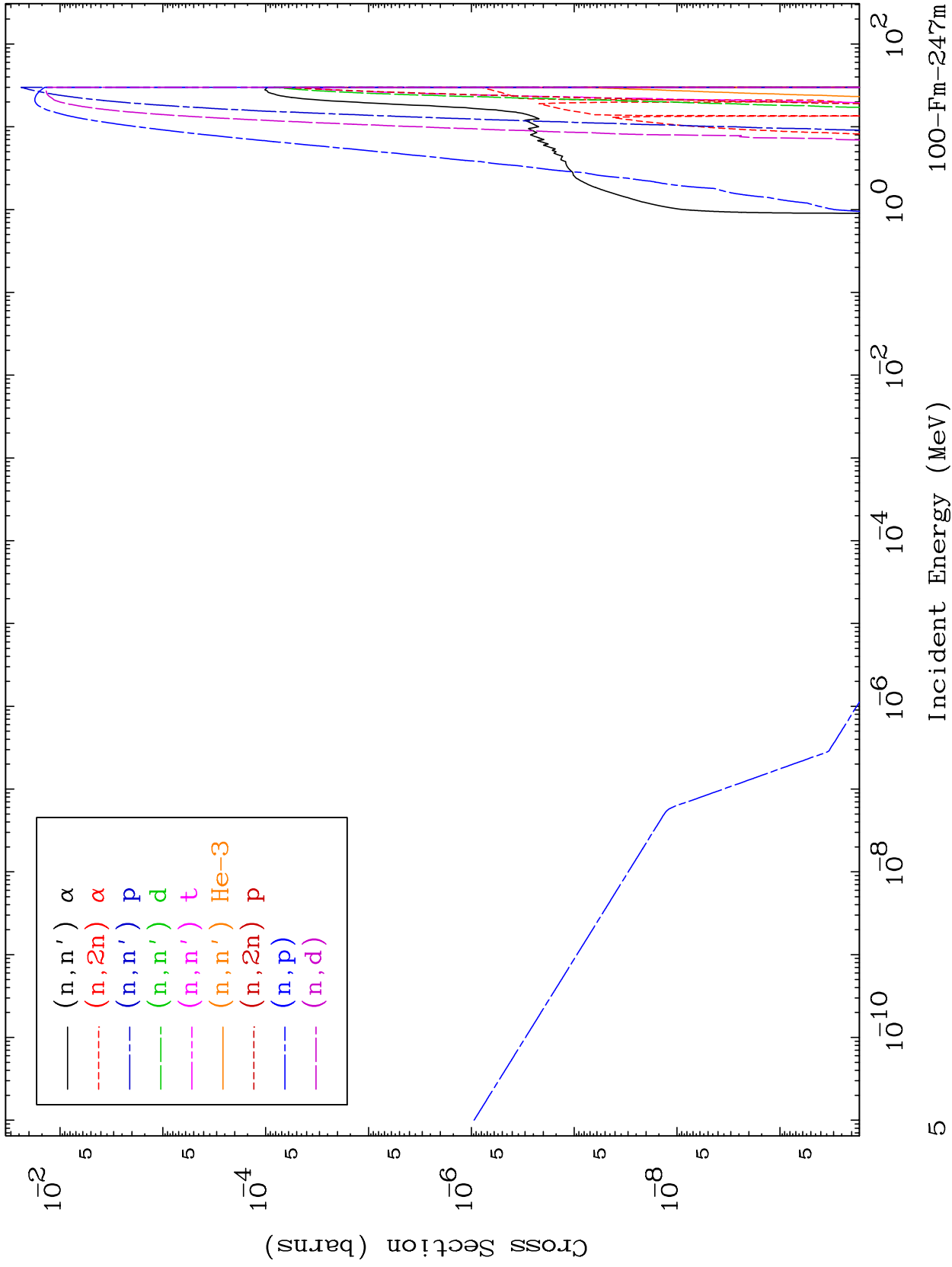
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MAT 9933

Charged Particle
293 Kelvin Cross Sections

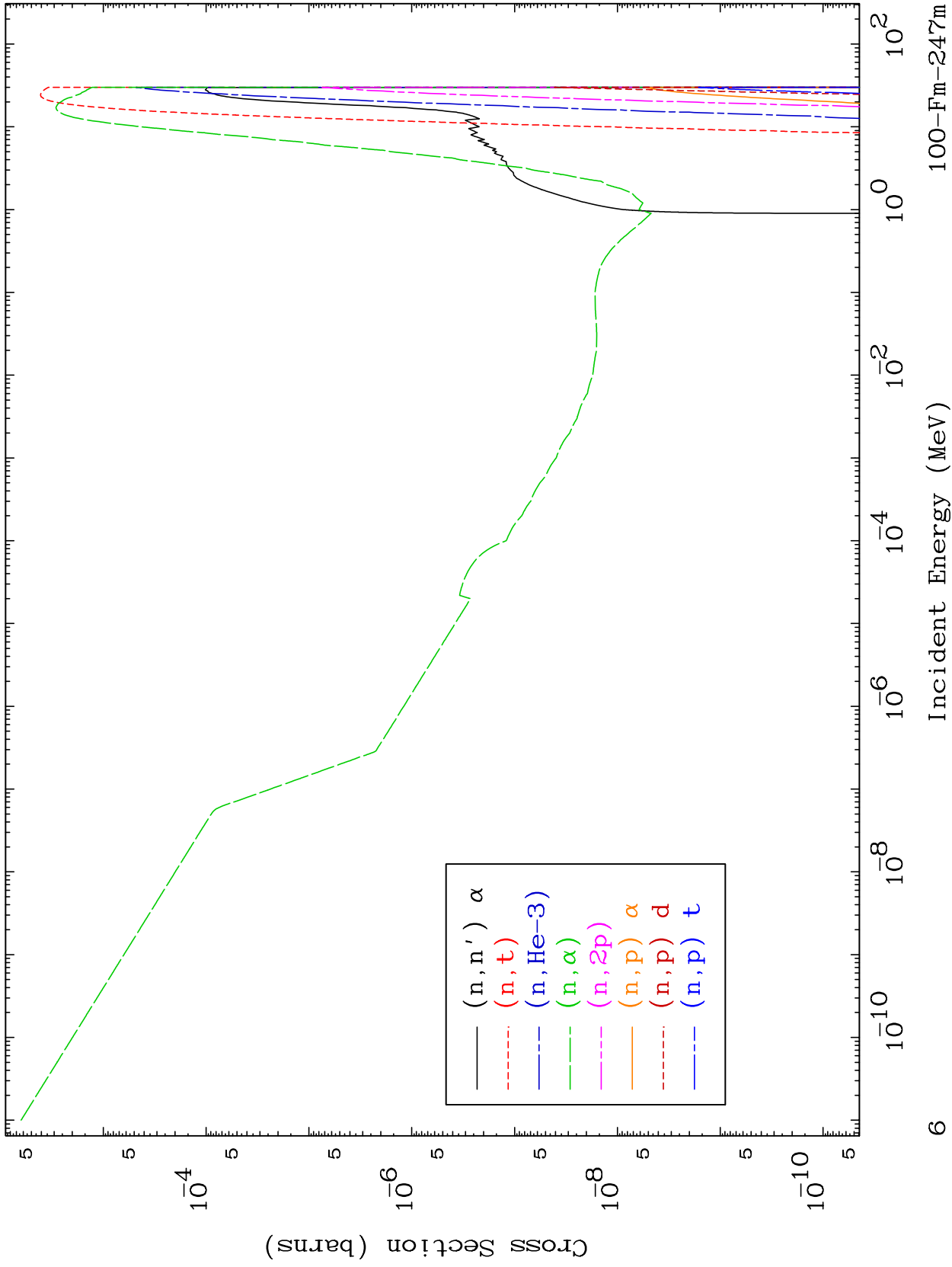
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Charged Particle
293 Kelvin Cross Sections

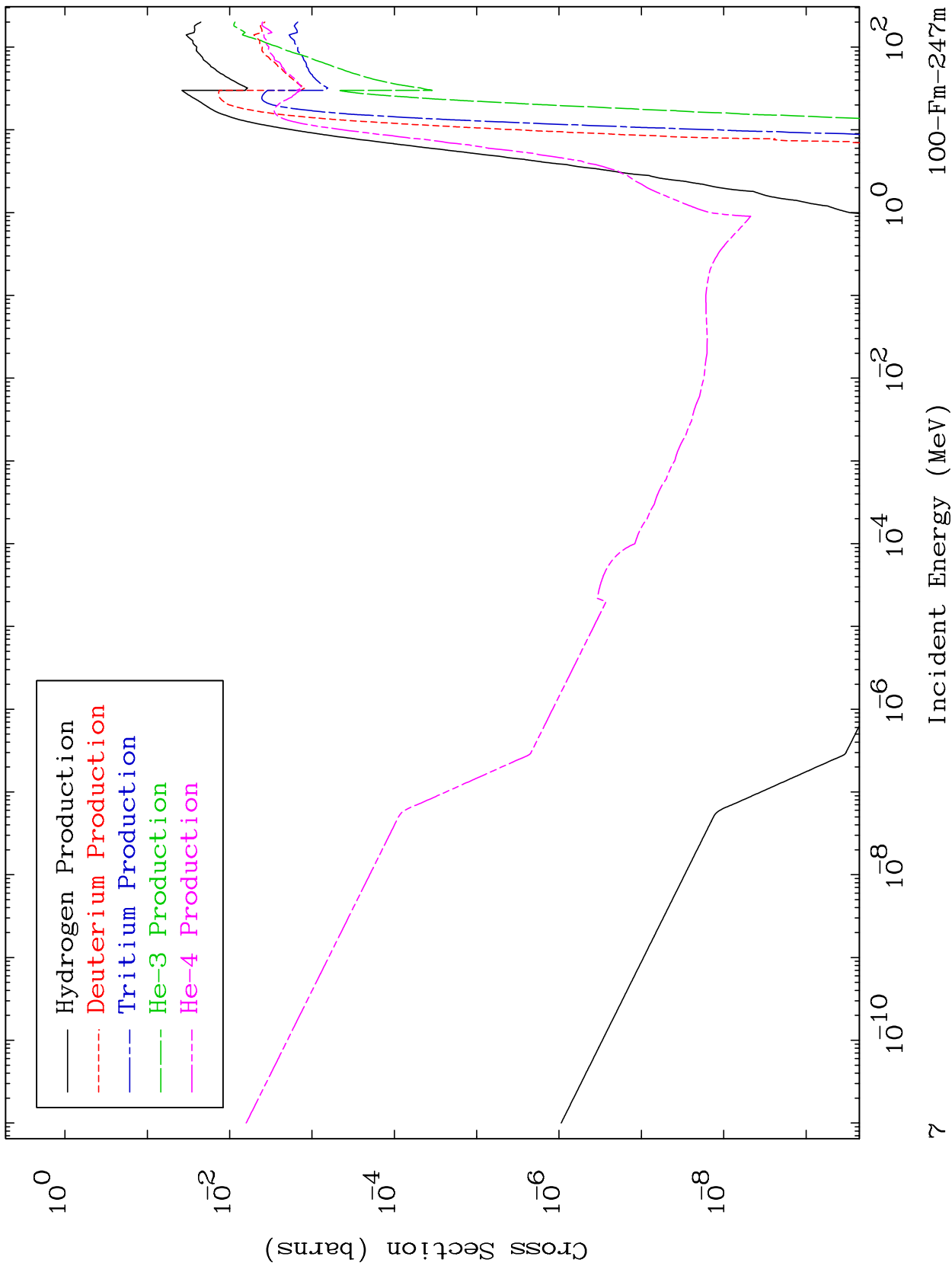
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Particle Production
293 Kelvin Cross Sections

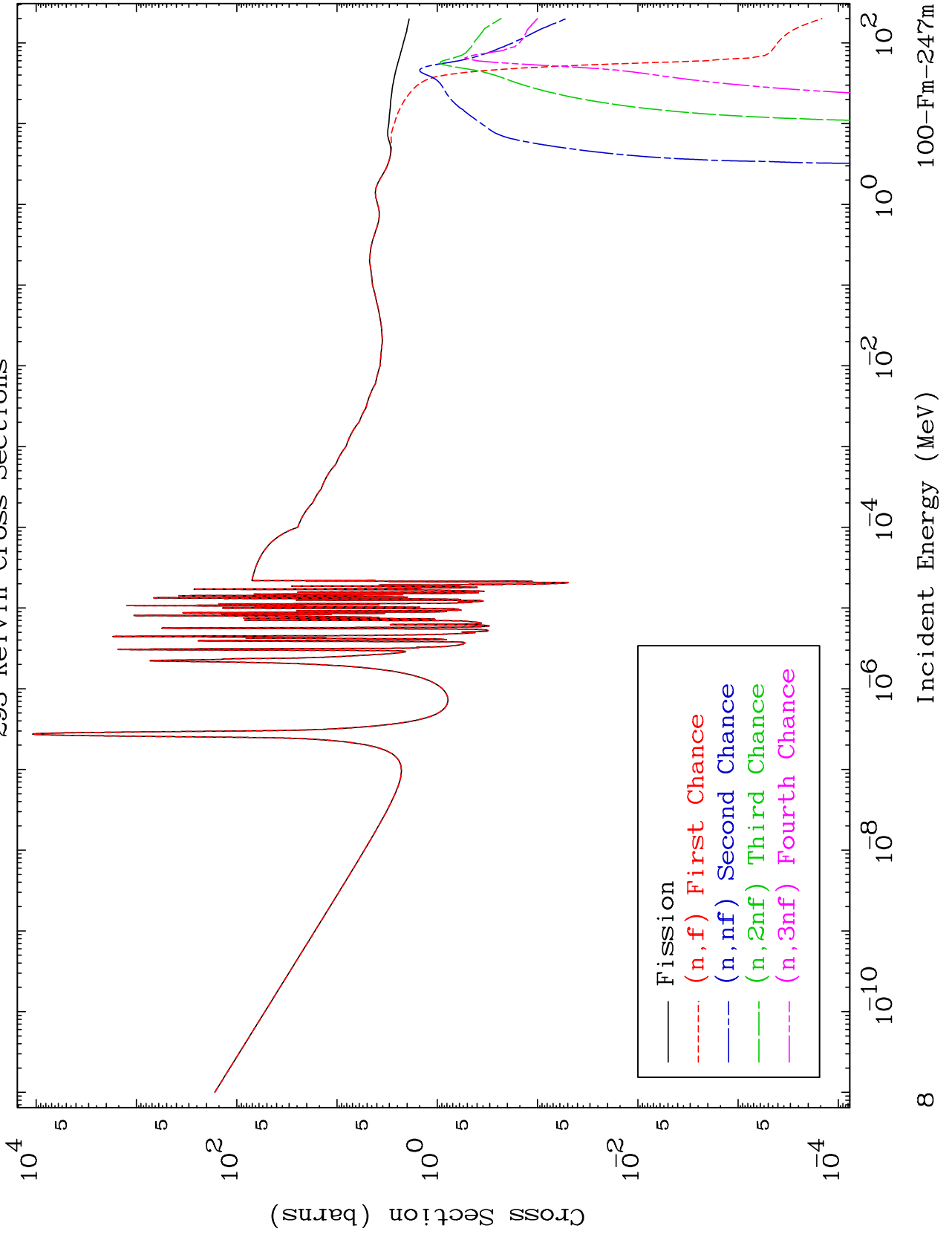
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Fission
293 Kelvin Cross Sections

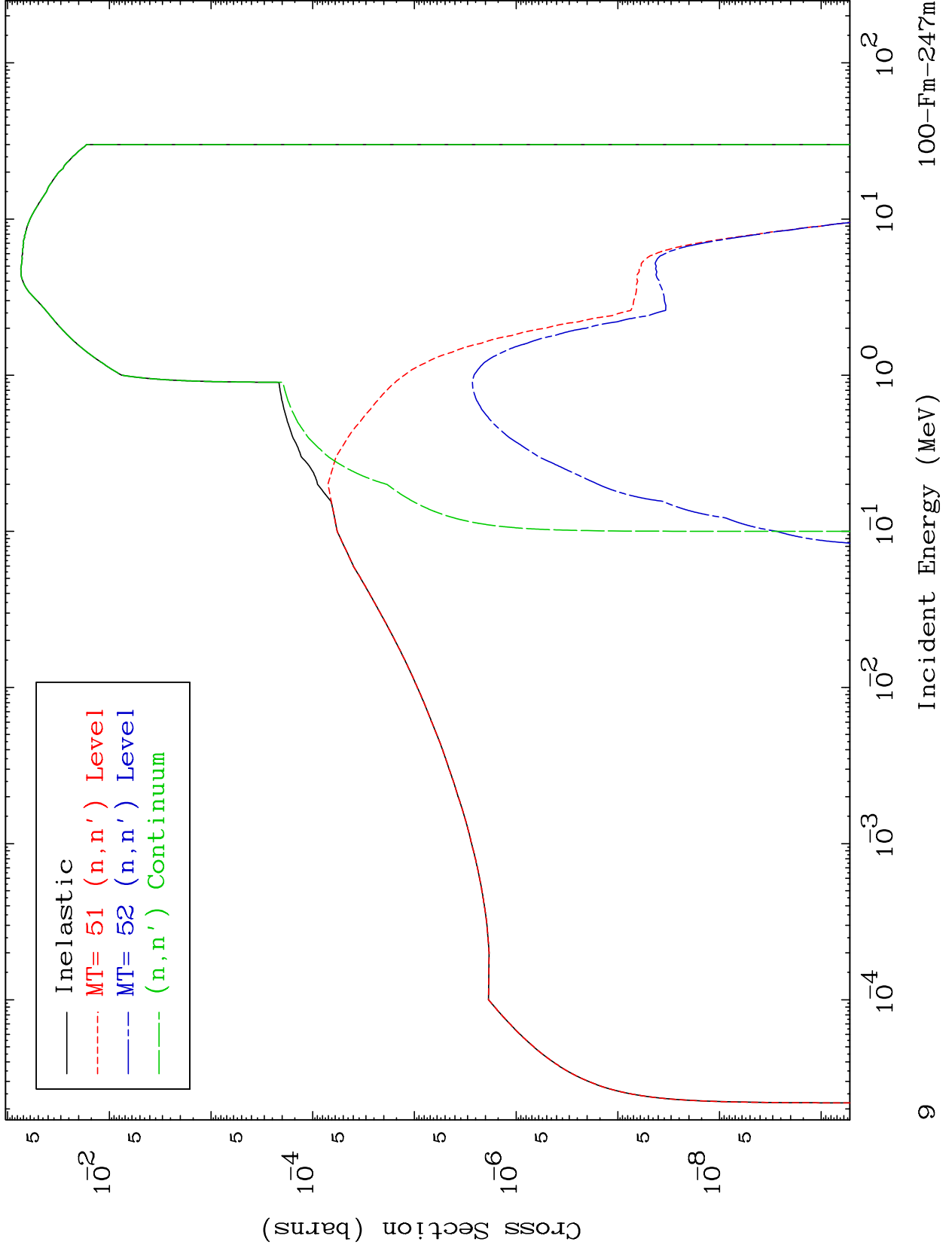
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(n,n') Levels
293 Kelvin Cross Sections

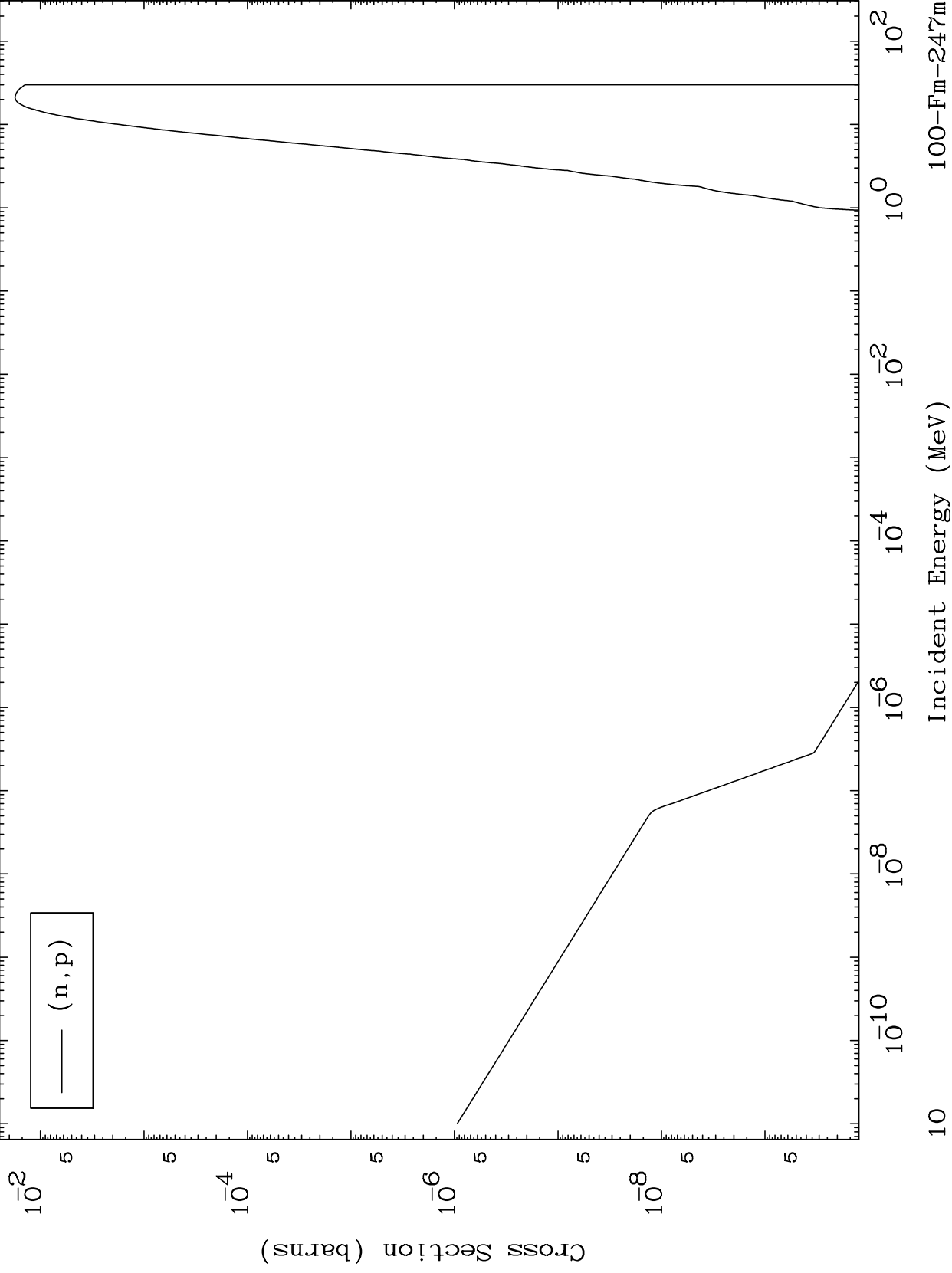
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(n,p) Levels
293 Kelvin Cross Sections

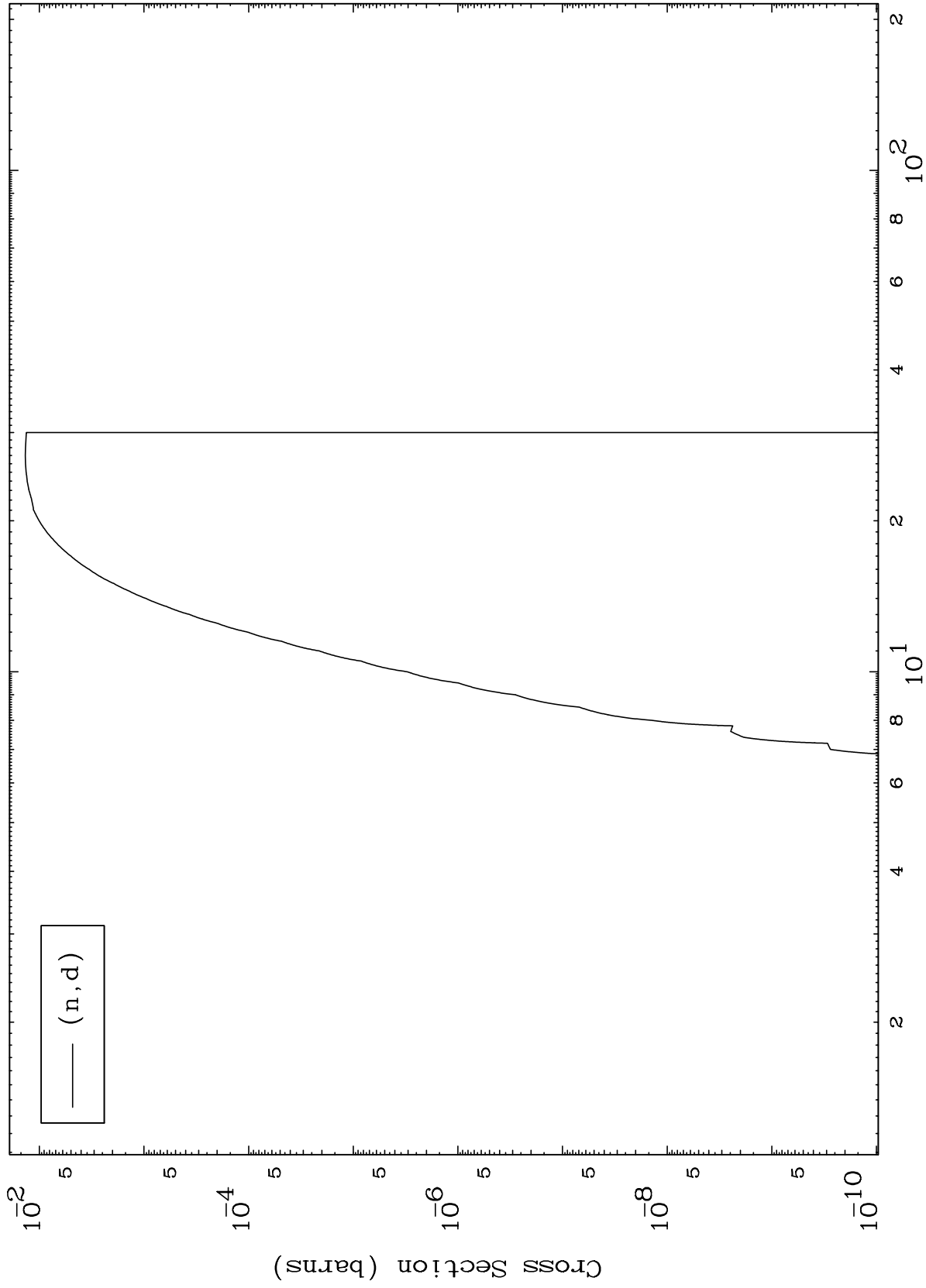
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(n,d) Levels
293 Kelvin Cross Sections

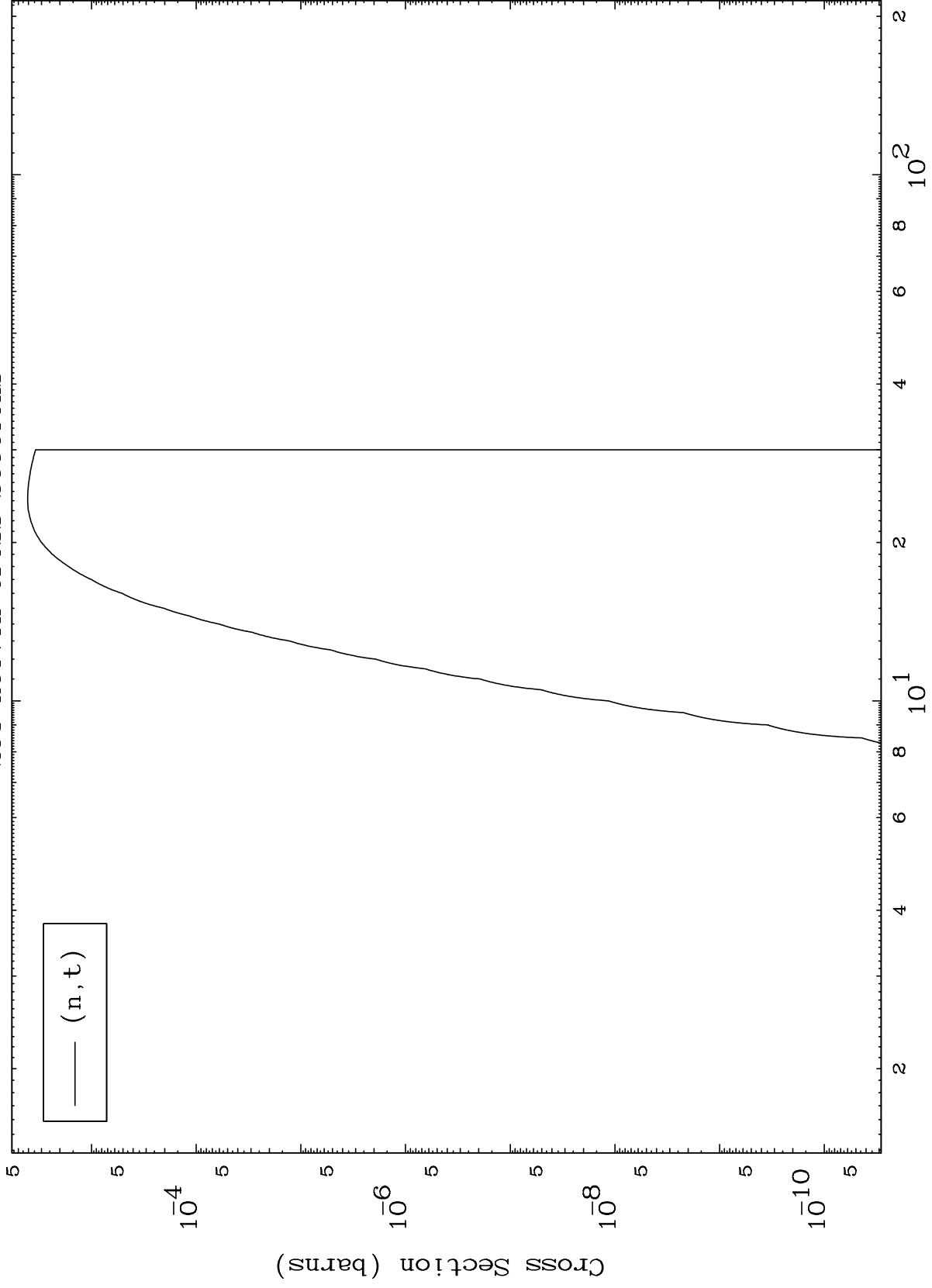
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(n,t) Levels
293 Kelvin Cross Sections

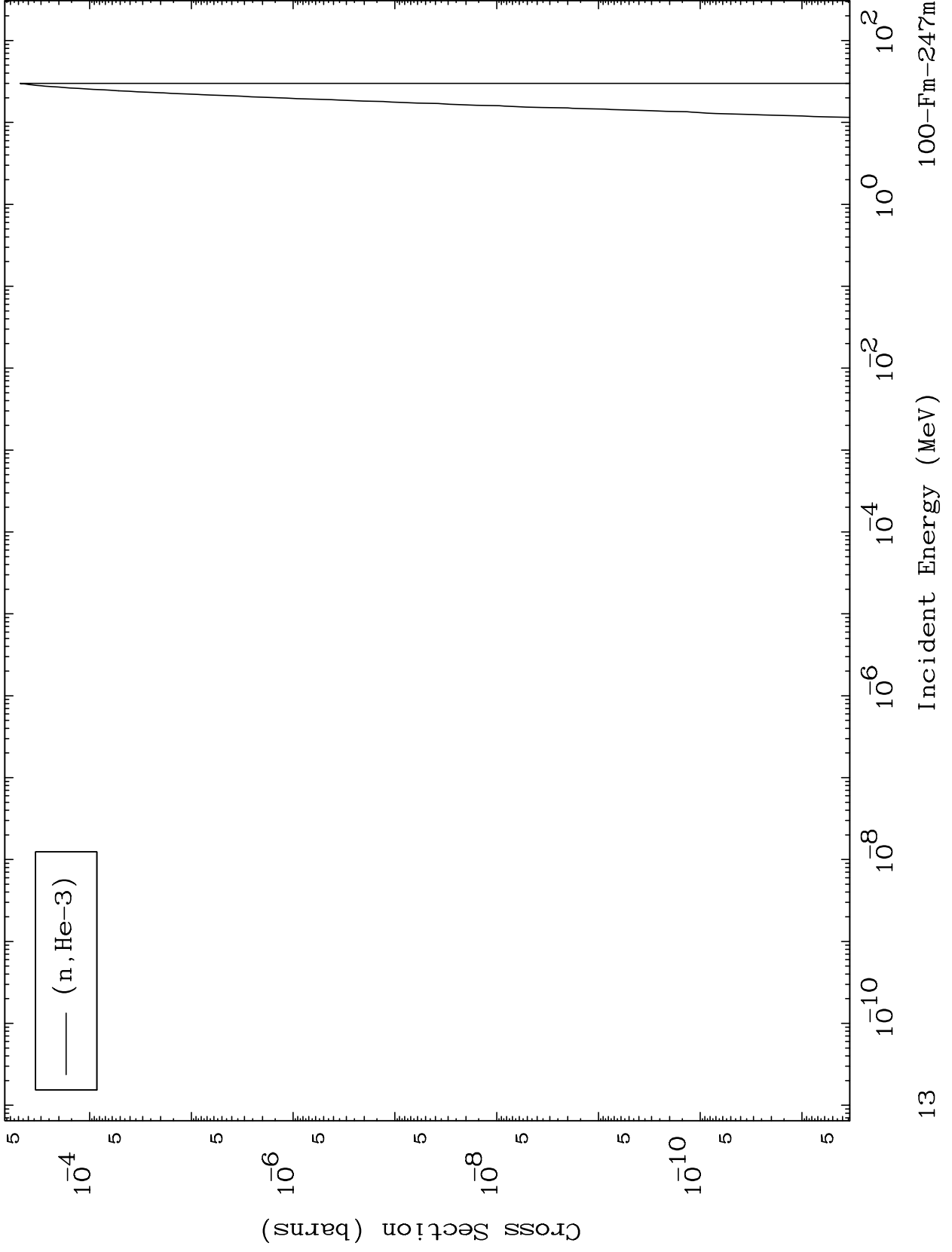
100-Fm-247m



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(n,He3) Levels
293 Kelvin Cross Sections

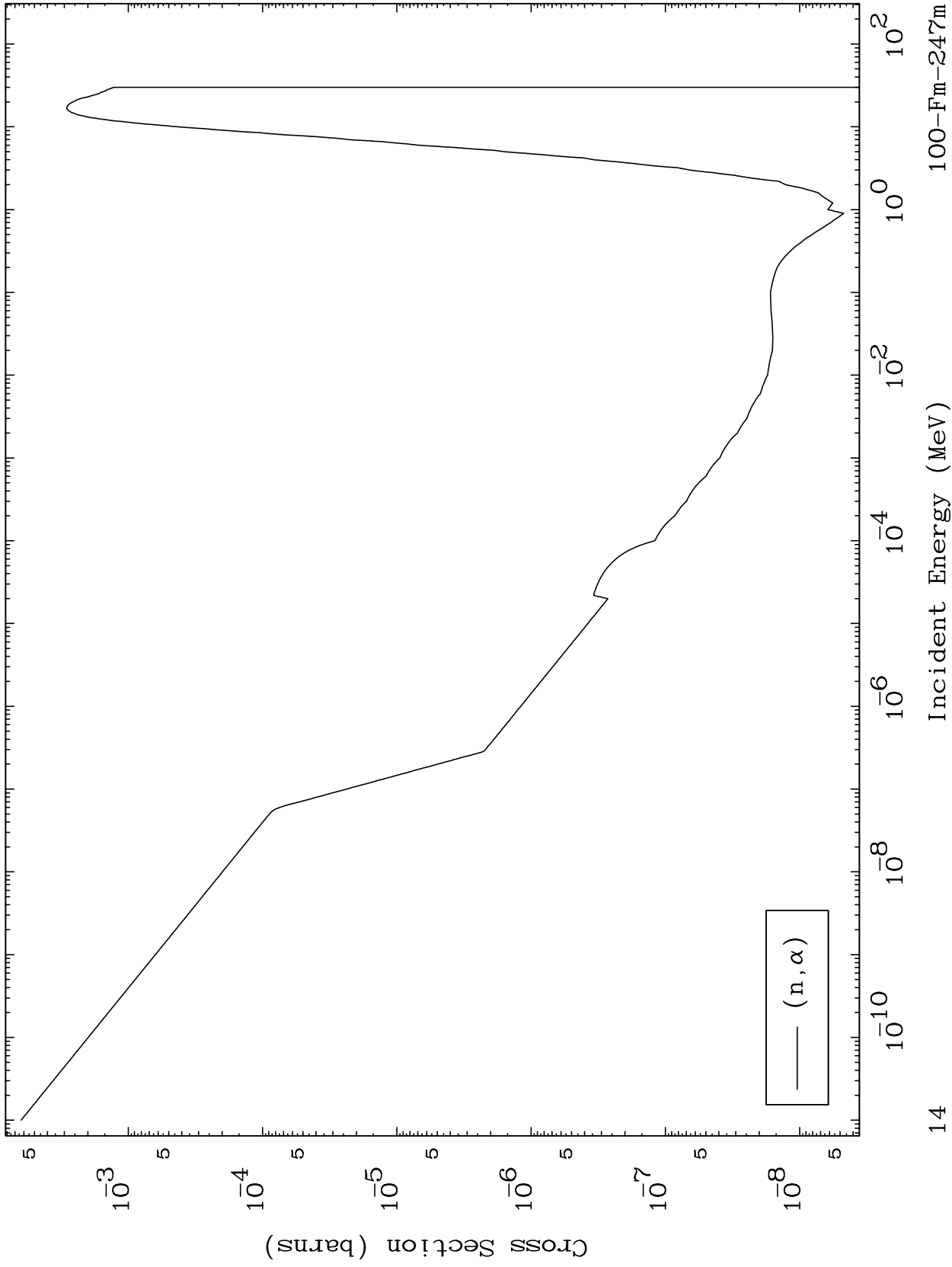
100-Fm-247m



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(n,α) Levels
293 Kelvin Cross Sections

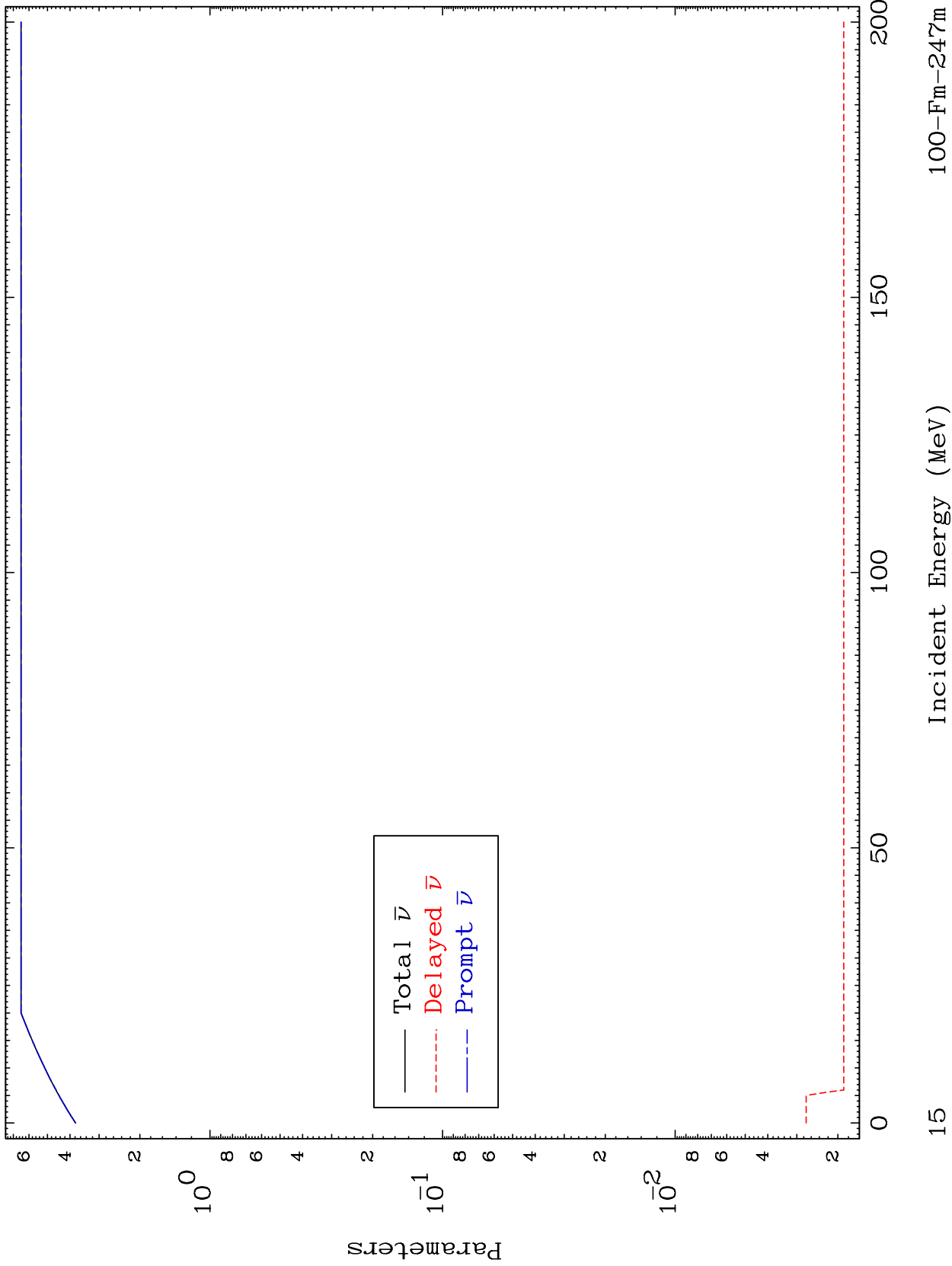
100-Fm-247m



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Energy Release
Parameters

100-Fm-247m



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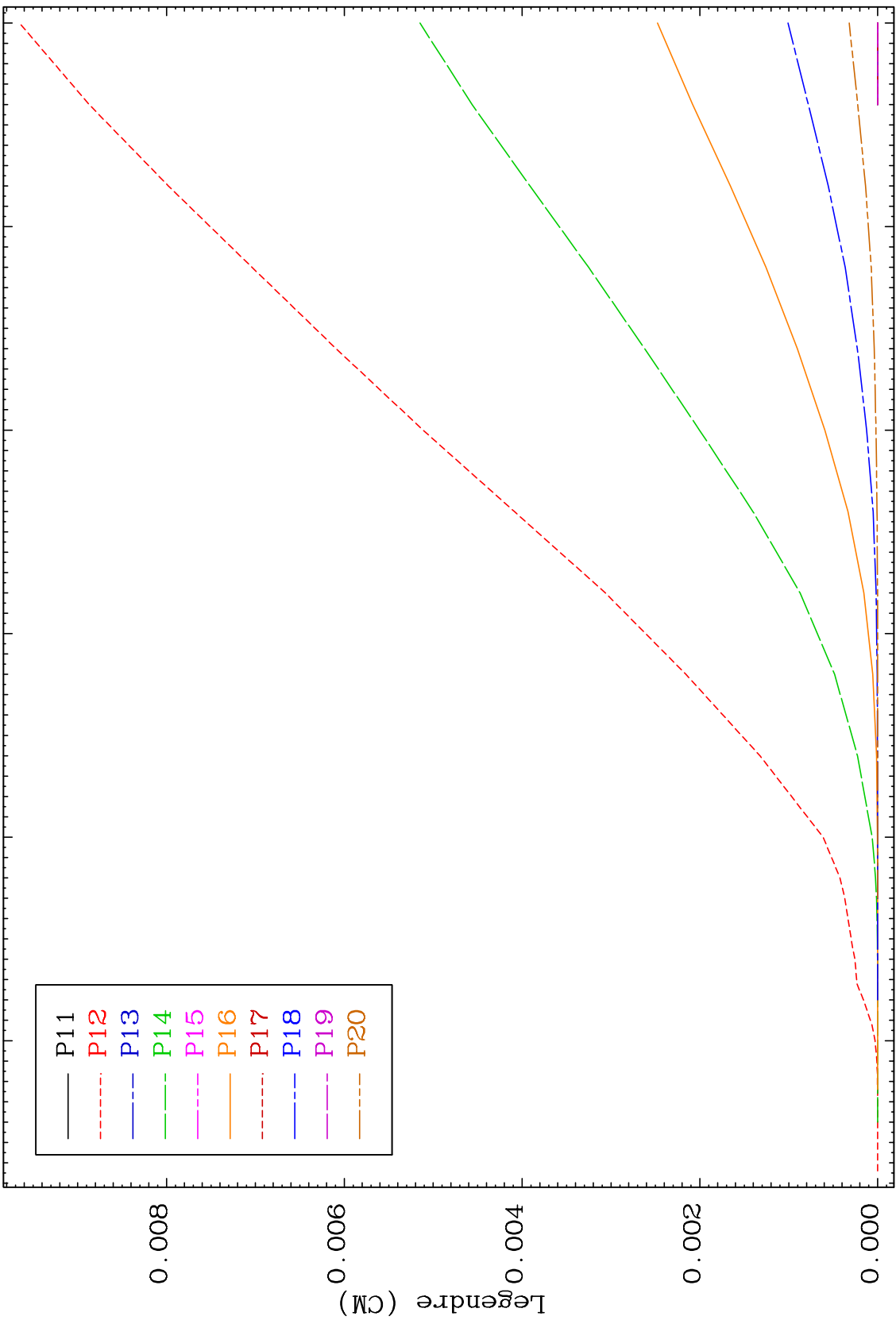
Incident Energy (MeV)

100-Fm-247m

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Elastic Legendre Coefficients

100-Fm-247m



17

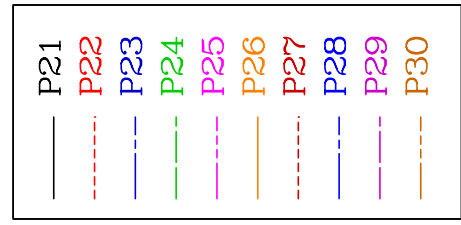
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Incident Energy (MeV)

MAT 9933

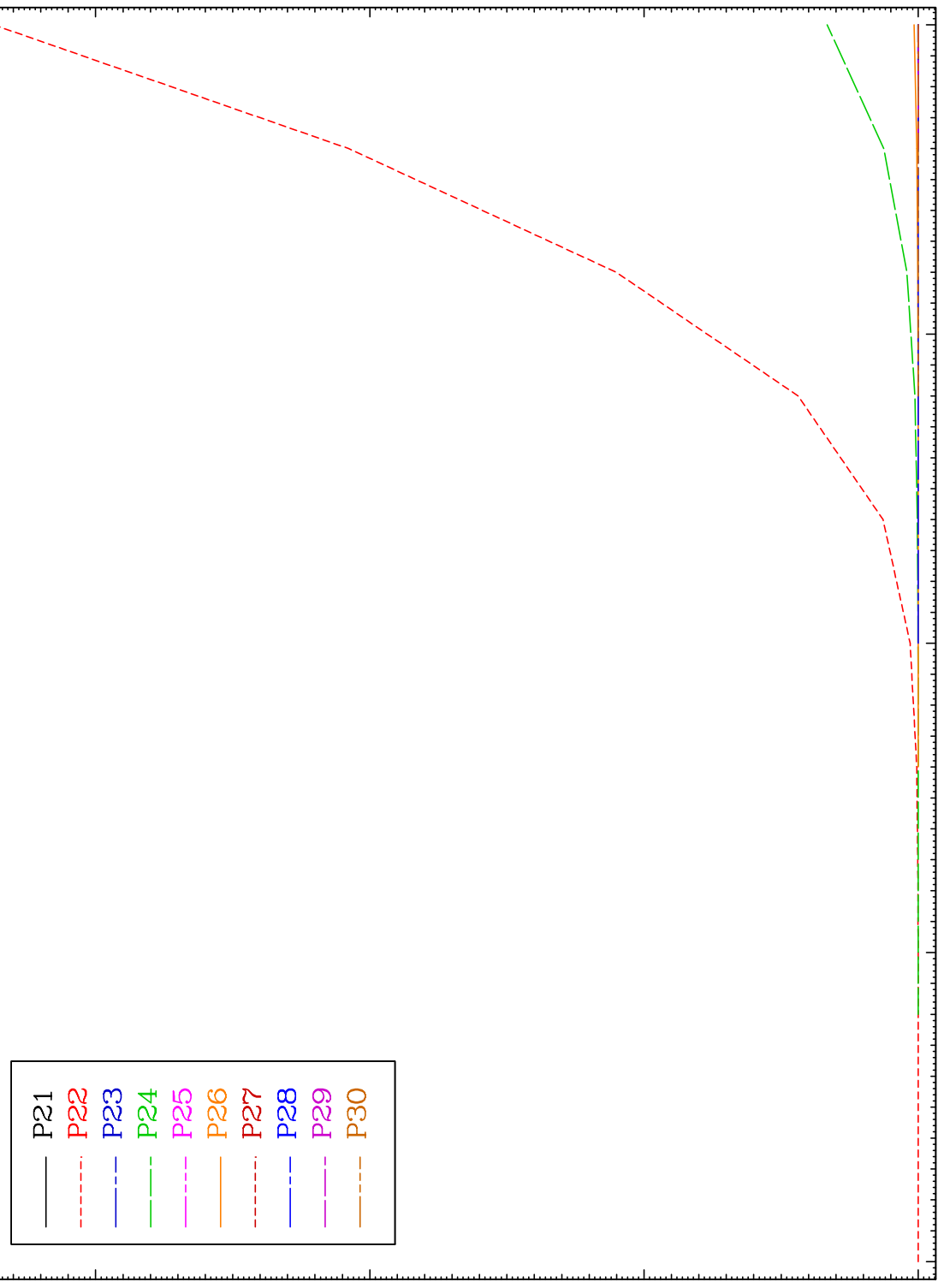
Elastic
Legendre Coefficients

100-Fm-247m



$\times 10^{-5}$

Legendre (CM)



18

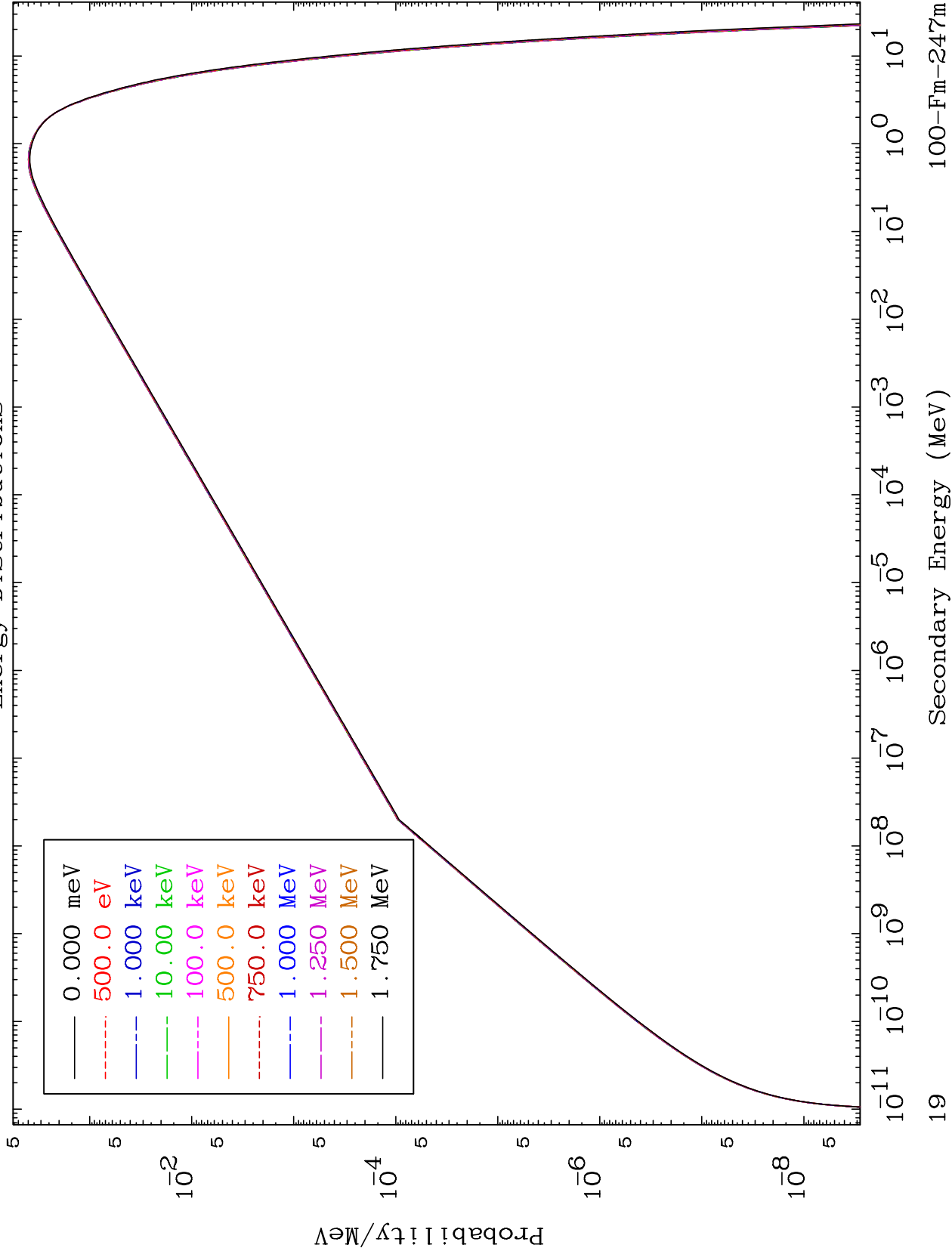
Incident Energy (MeV)

100-Fm-247m

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Fission Energy Distributions

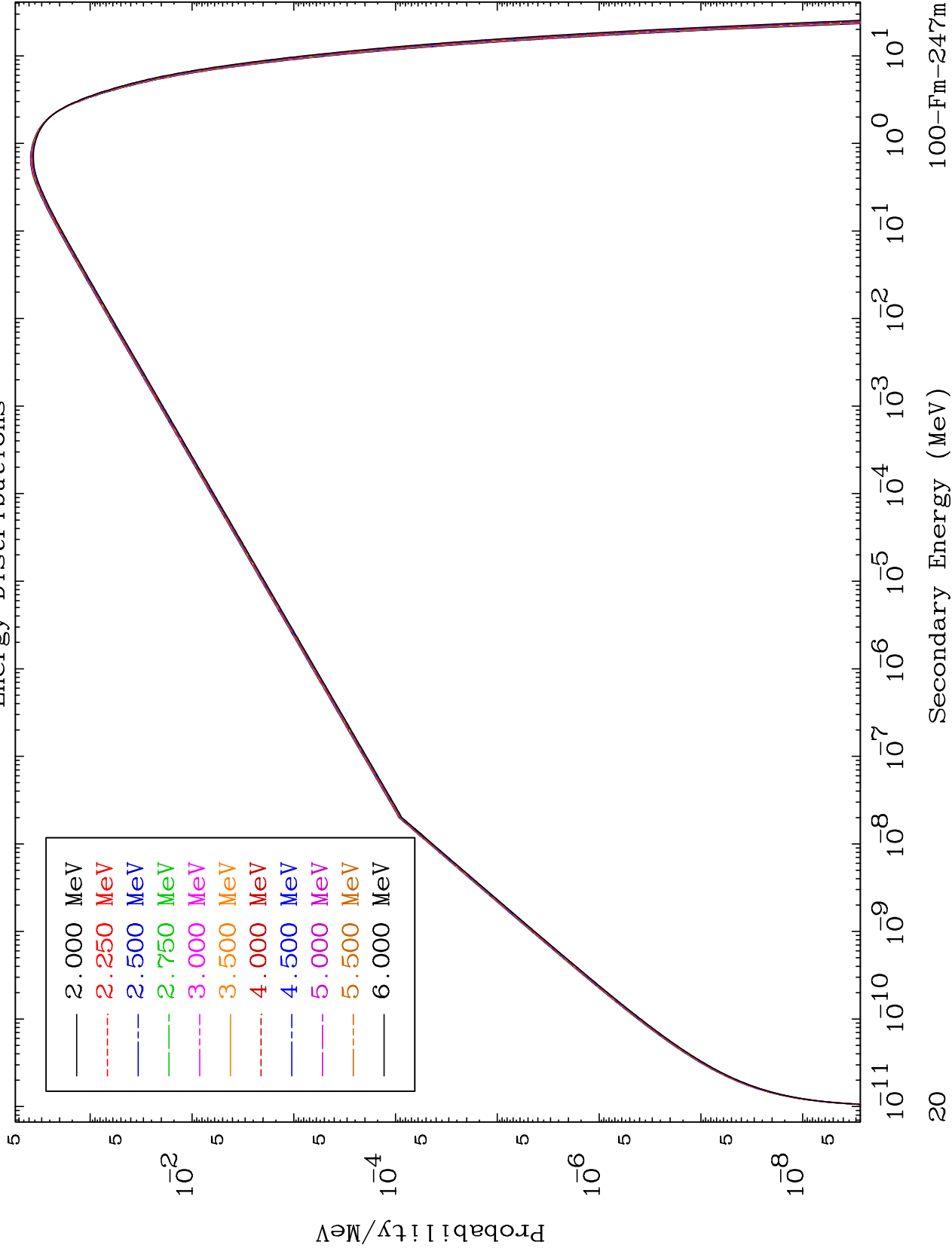
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Fission Energy Distributions

100-Fm-247m



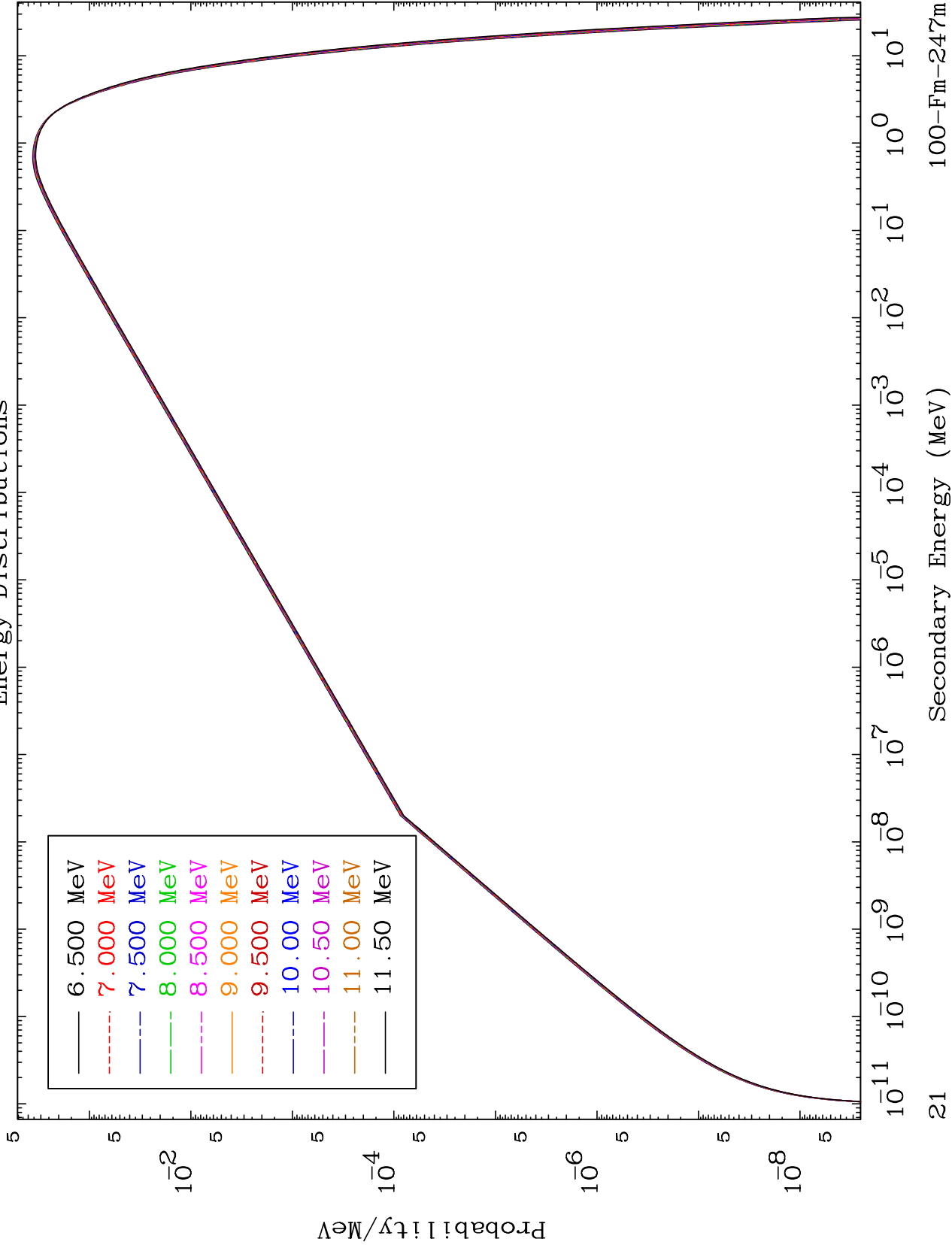
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20

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Fission Energy Distributions

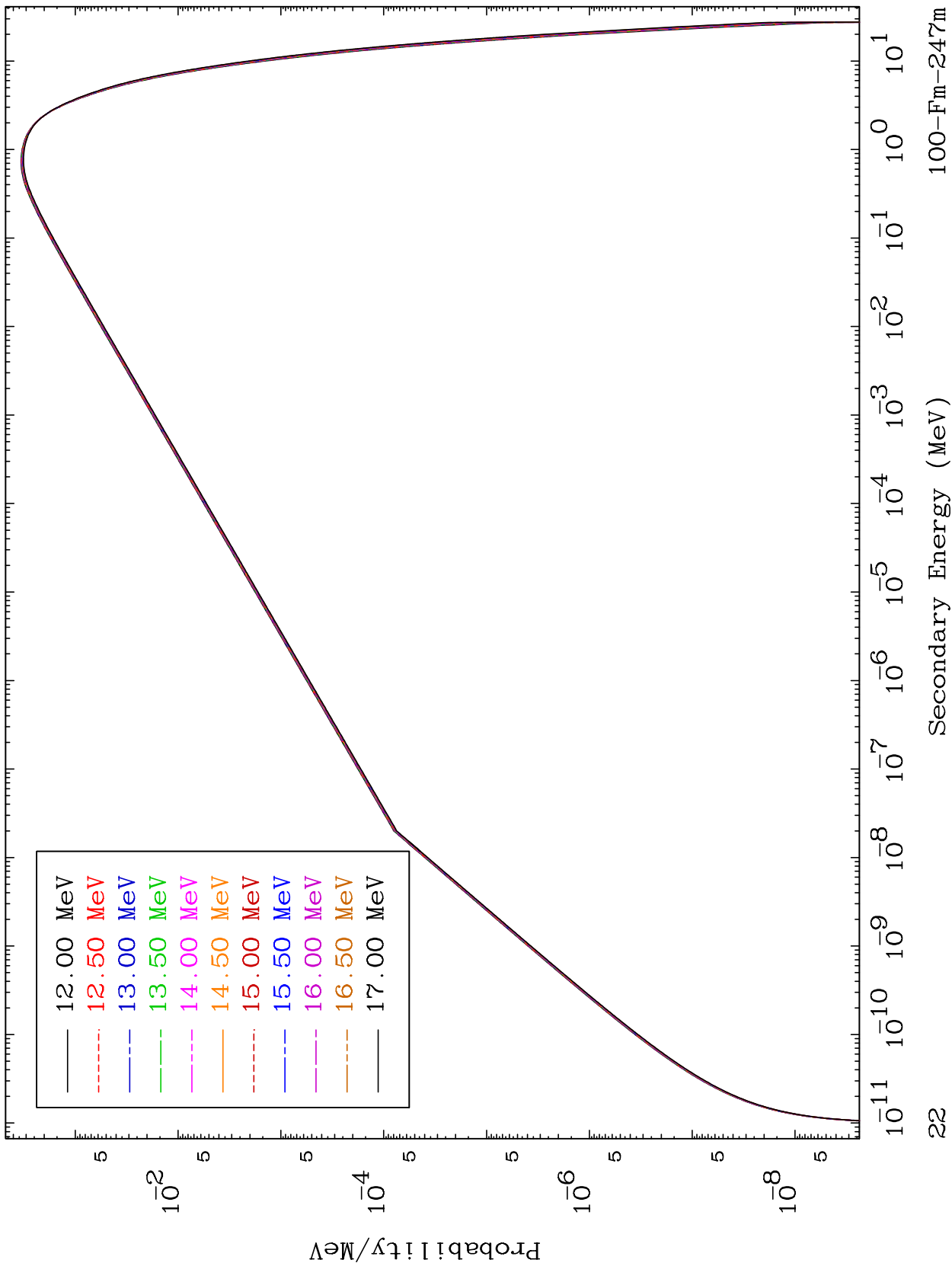
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Fission Energy Distributions

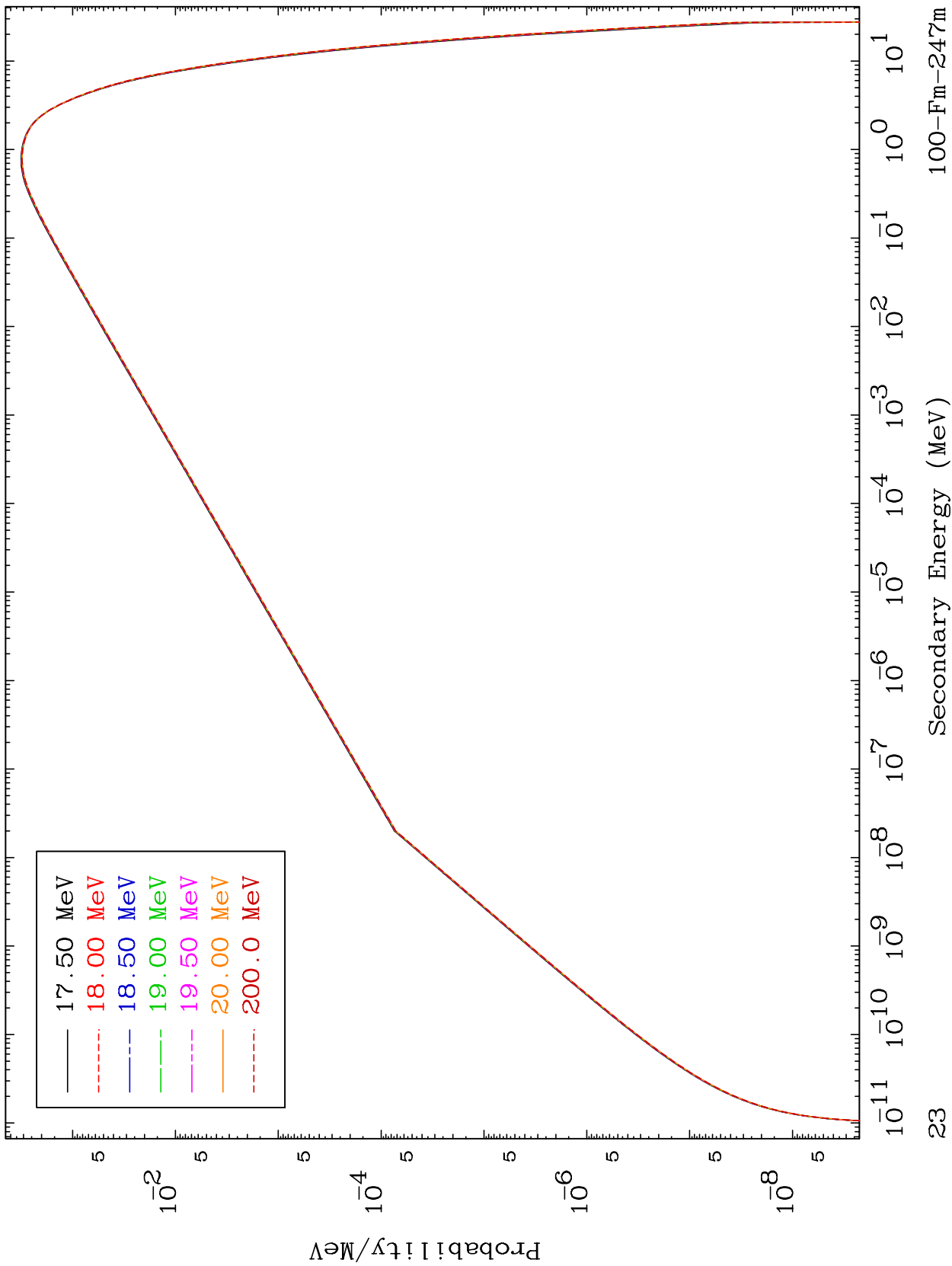
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Fission Energy Distributions

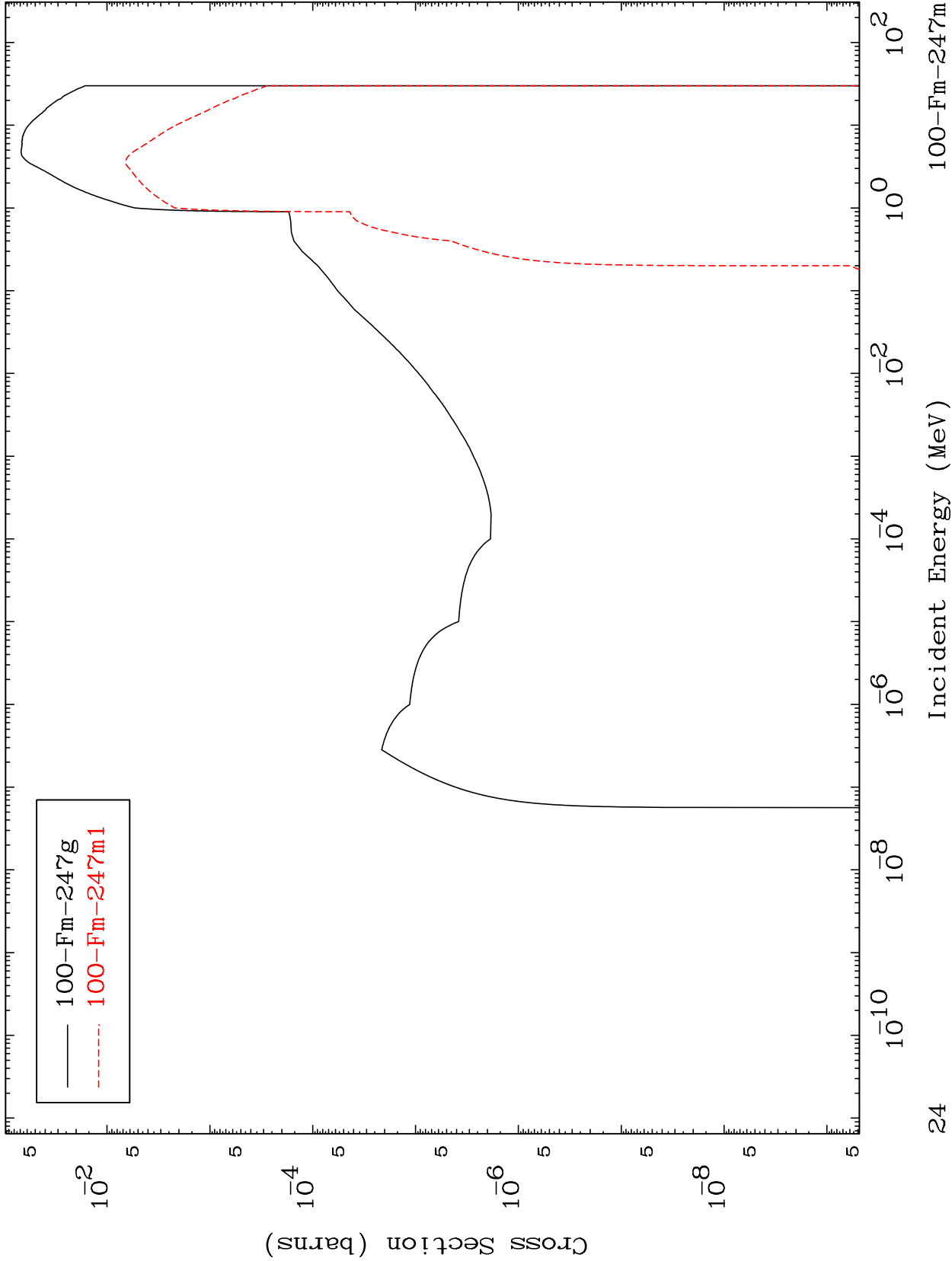
100-Fm-247m



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Inelastic
Radionuclide Production Cross Section

100-Fm-247m



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