

Program EVALPLOT
(Version 2021-1)

by

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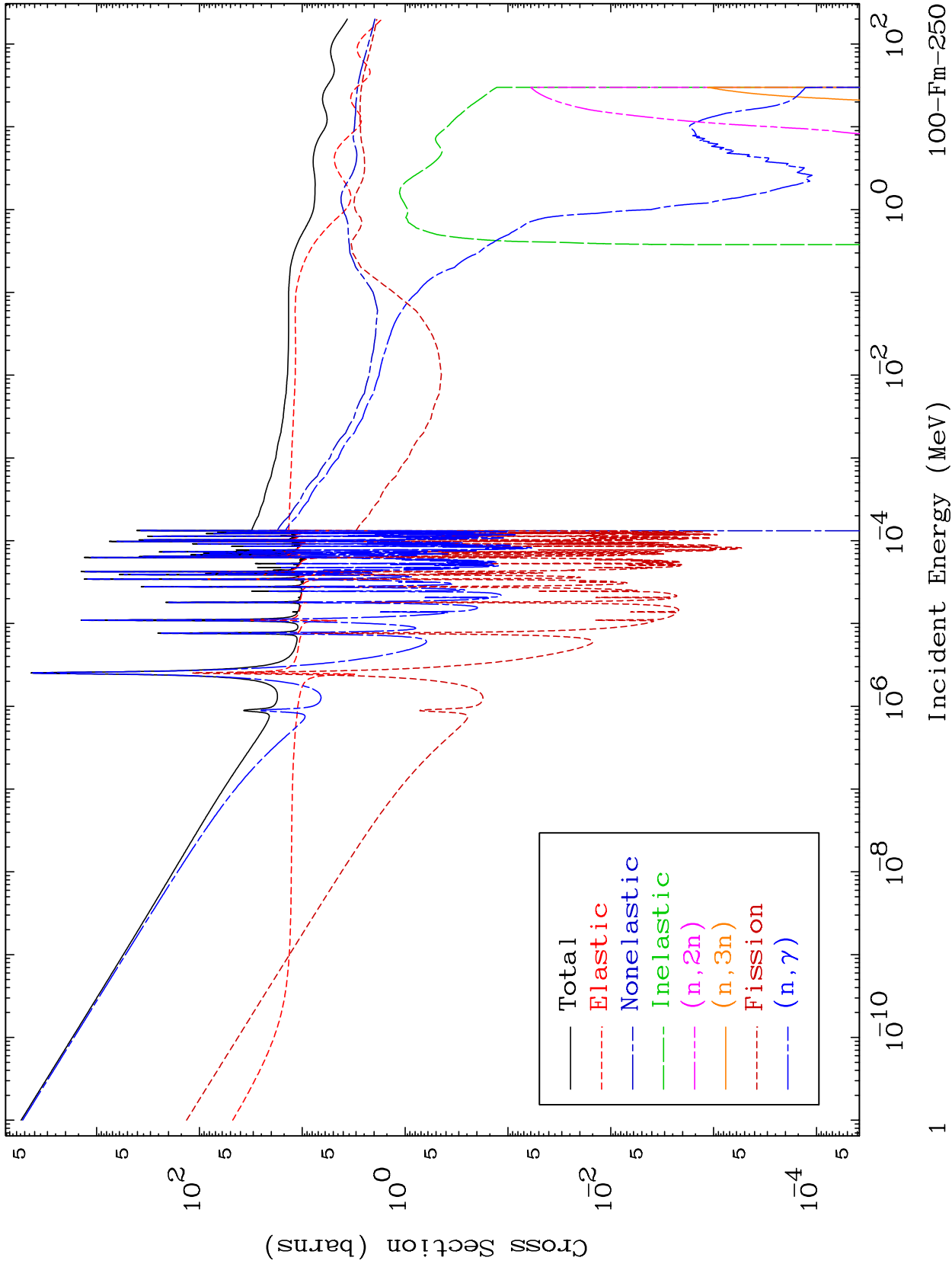
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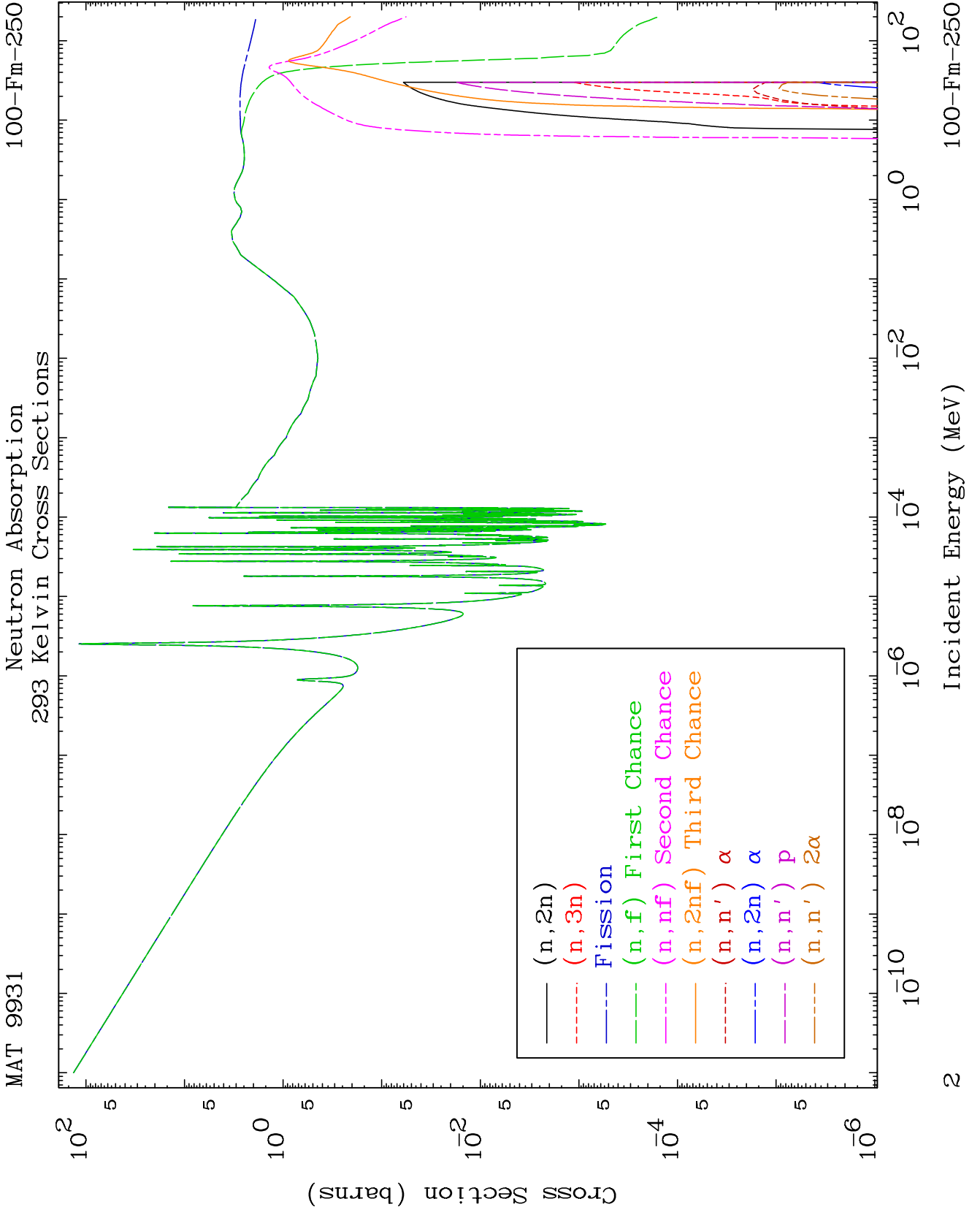
Press Mouse Button to Start

MAT 9931

Neutron Major
293 Kelvin Cross Sections

100-Fm-250

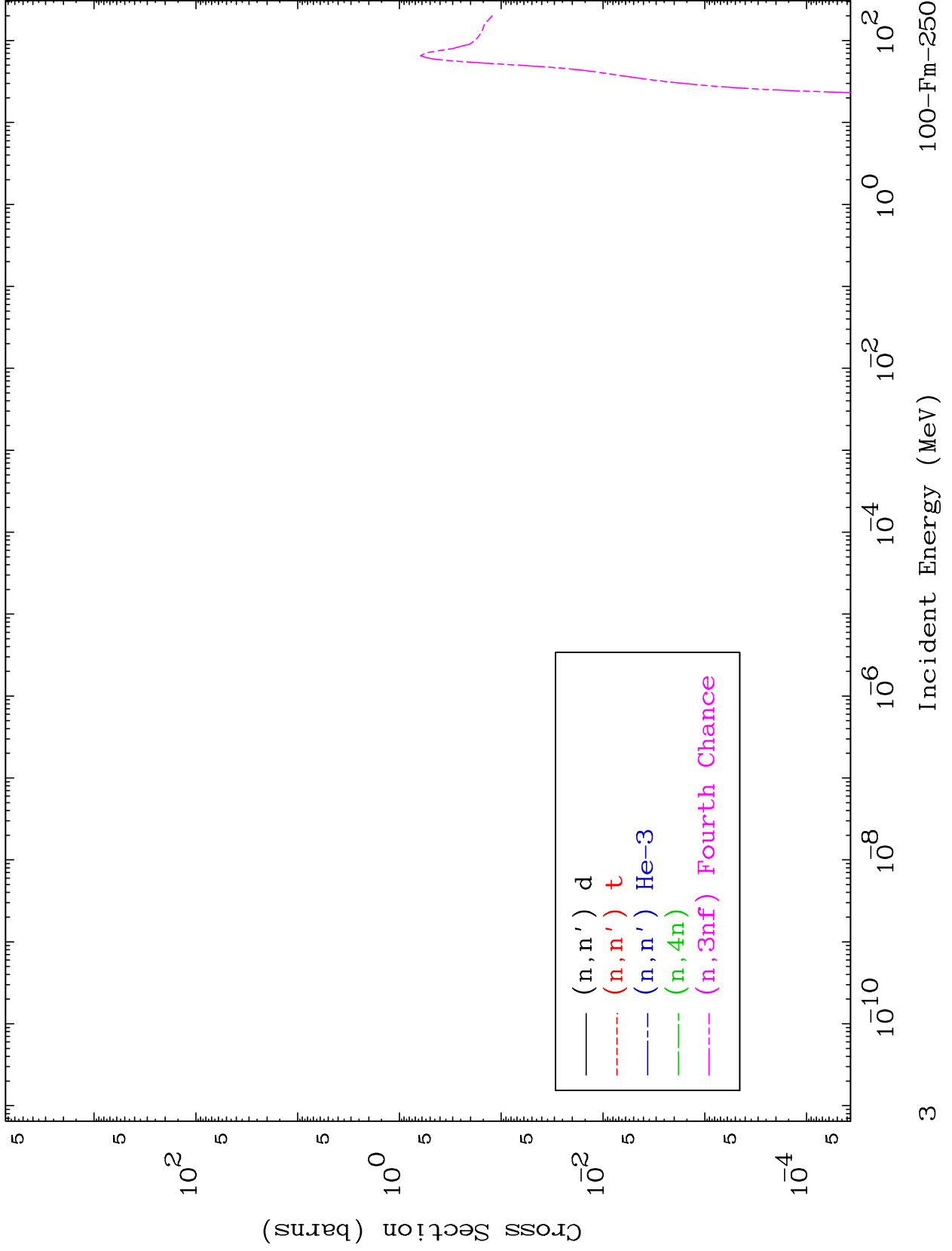




MAT 9931

Neutron Absorption
293 Kelvin Cross Sections

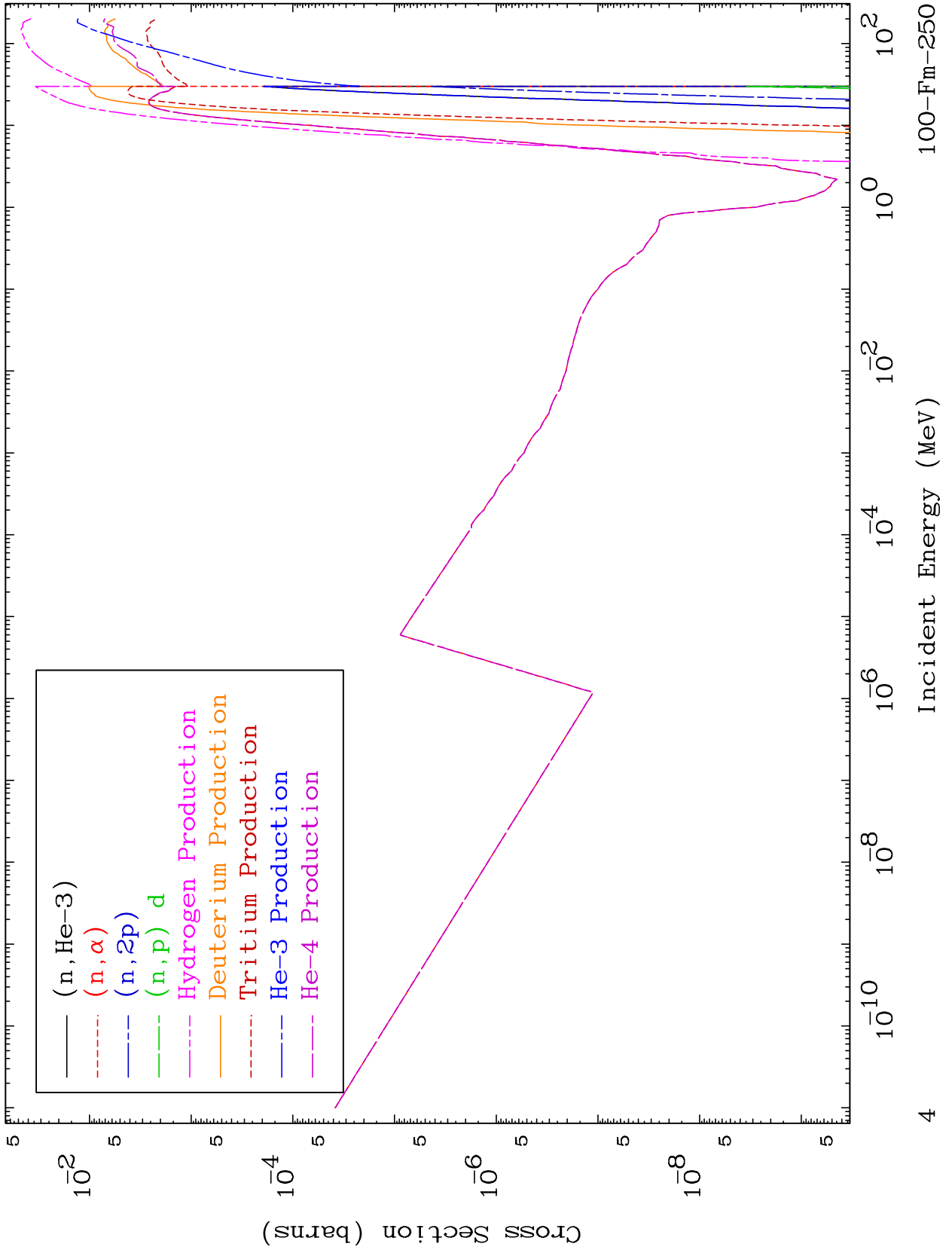
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MAT 9931

Neutron Absorption
293 Kelvin Cross Sections

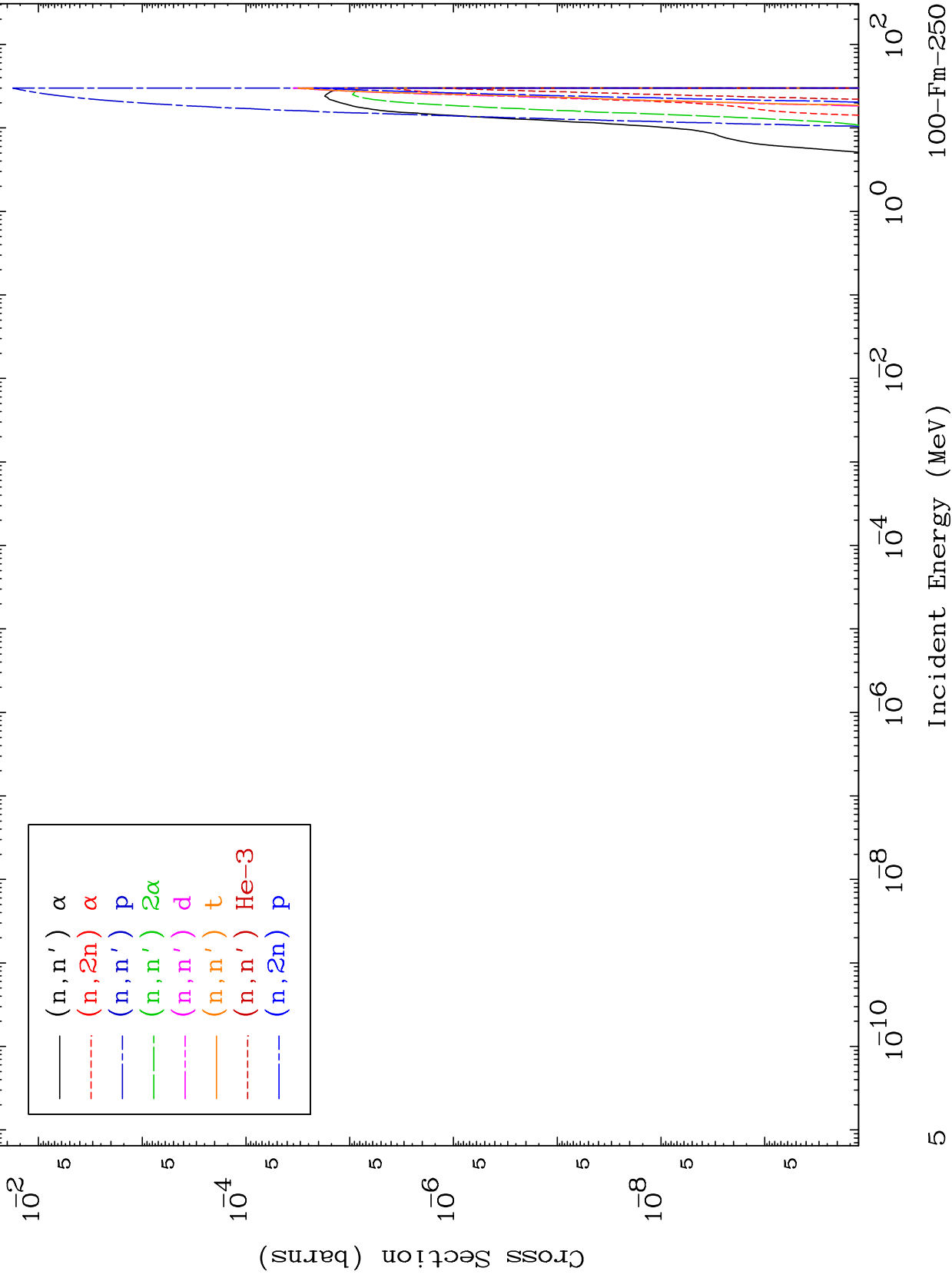
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MAT 9931

Charged Particle
293 Kelvin Cross Sections

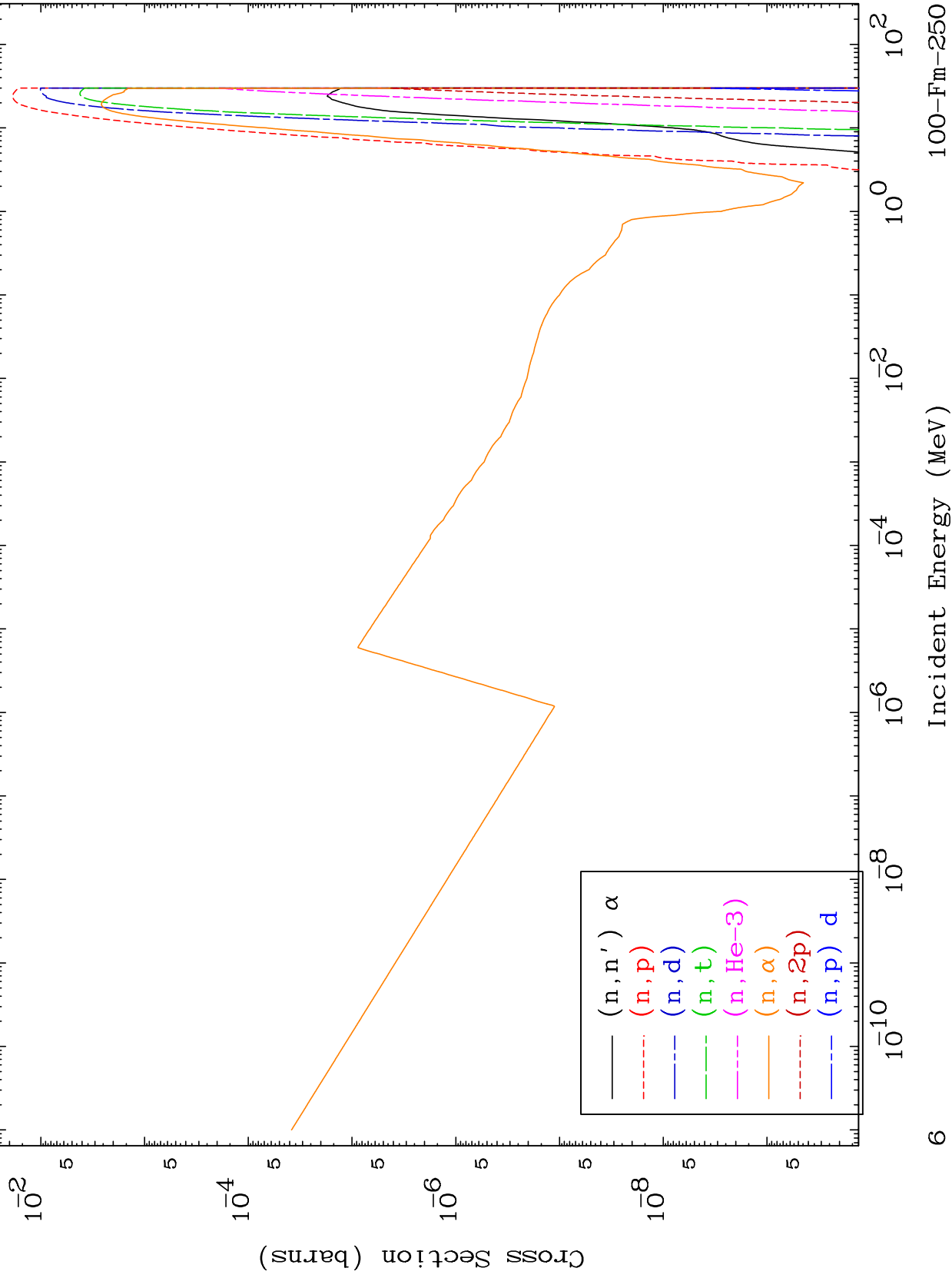
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MAT 9931

Charged Particle
293 Kelvin Cross Sections

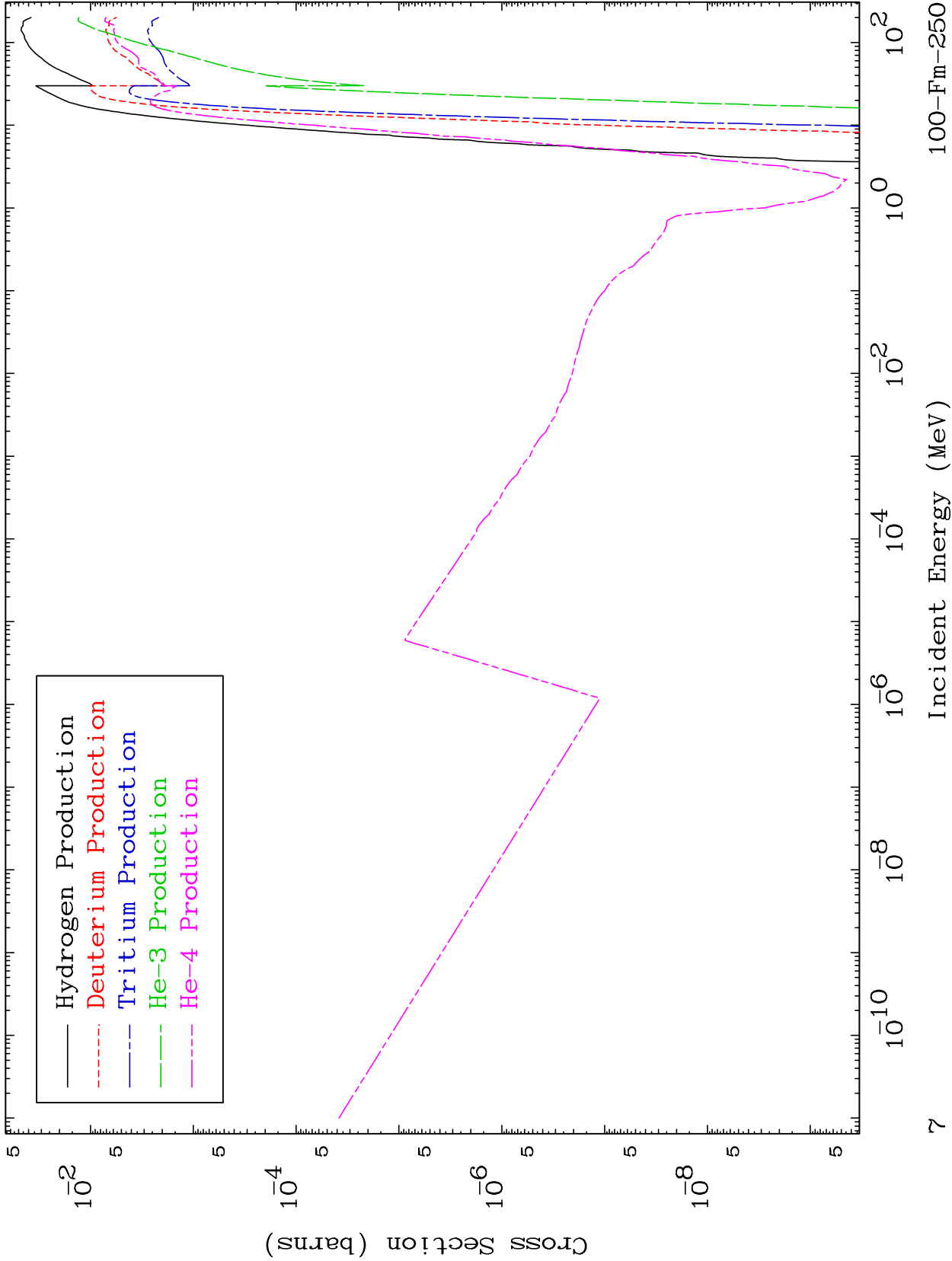
100-Fm-250



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Particle Production
293 Kelvin Cross Sections

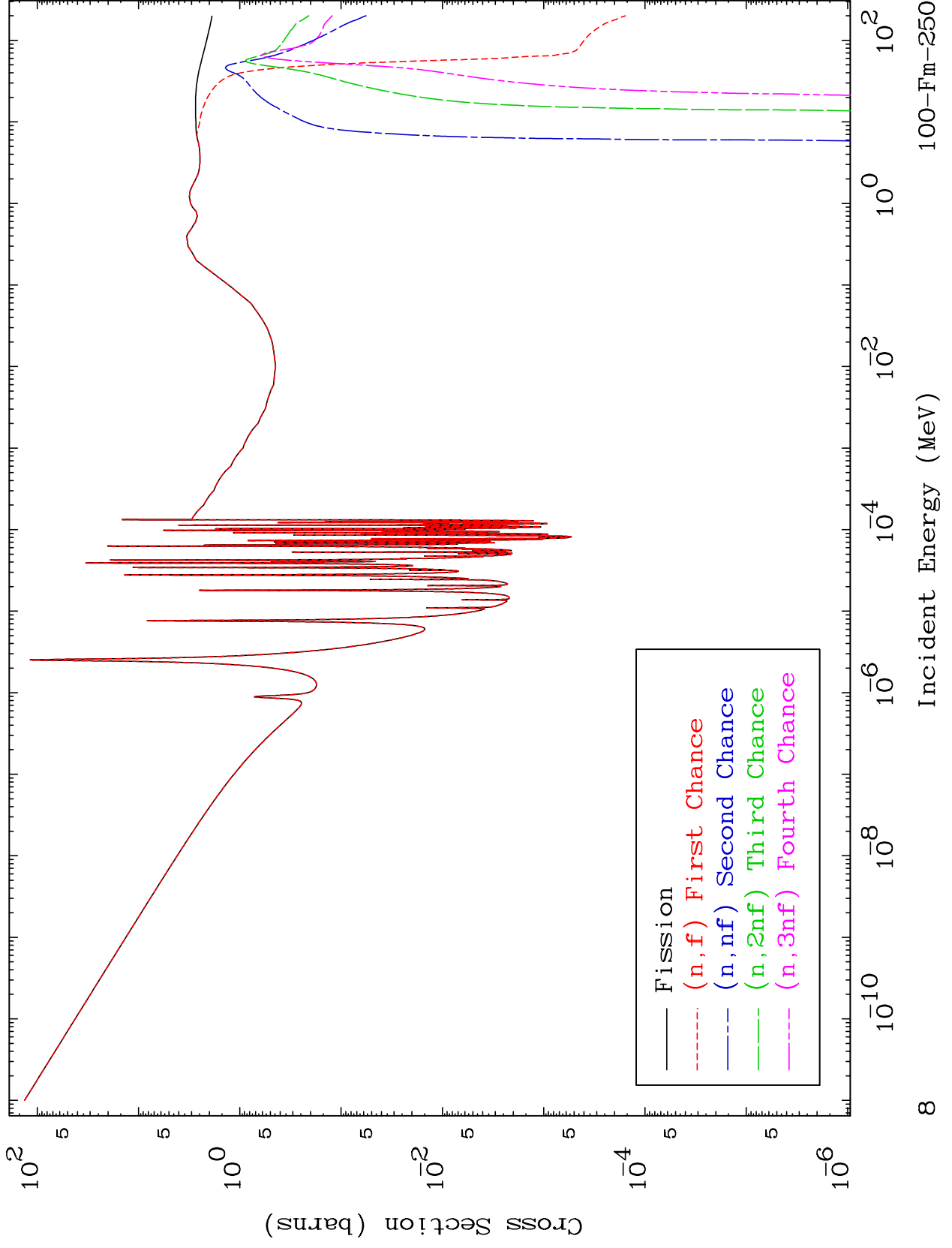
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MAT 9931

Fission
293 Kelvin Cross Sections

100-Fm-250

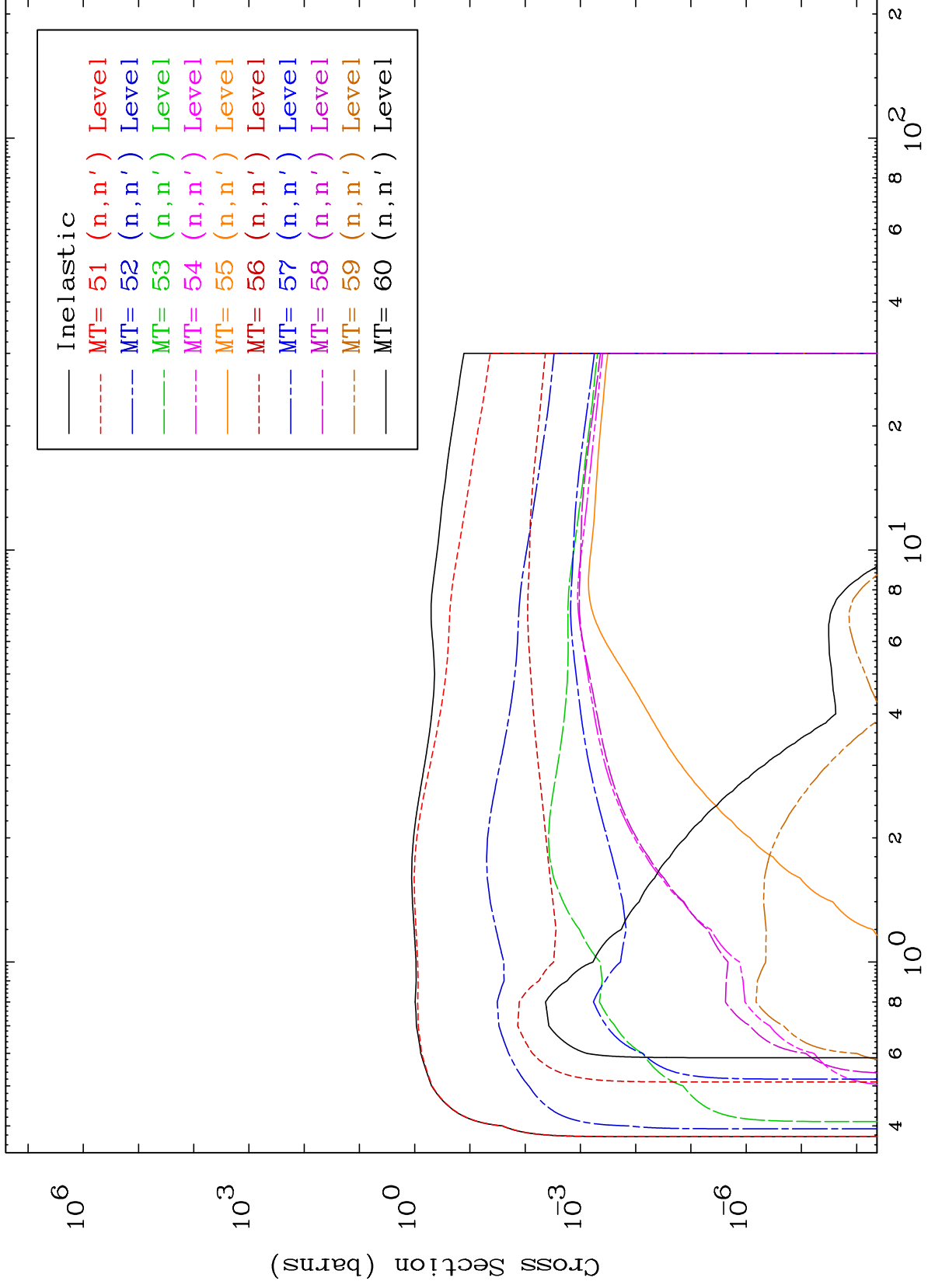


MAT 9931

(n,n') Levels

100-Fm-250

293 Kelvin Cross Sections



9

Incident Energy (MeV)

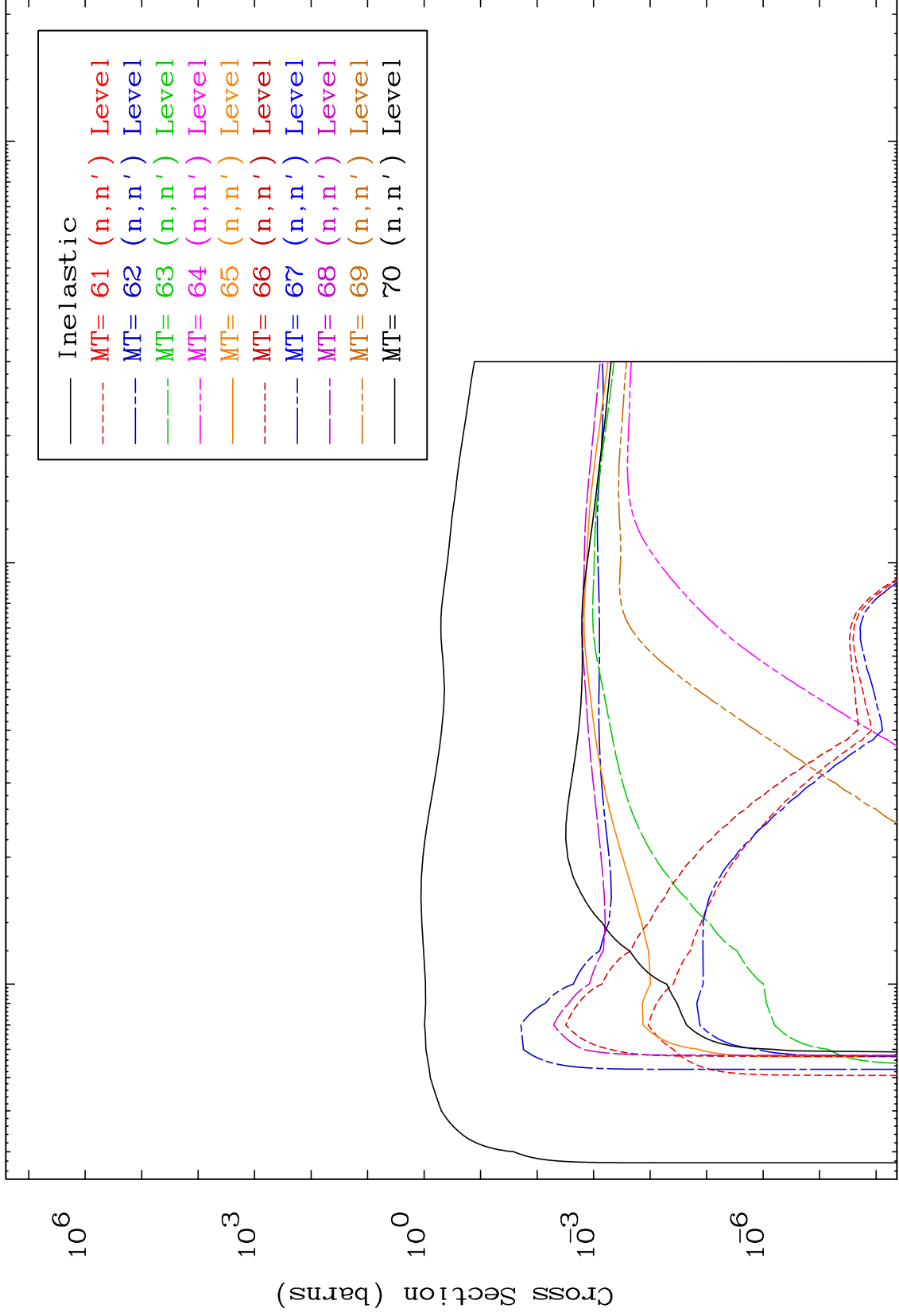
100-Fm-250

MAT 9931

(n,n') Levels

100-Fm-250

293 Kelvin Cross Sections



10

Incident Energy (MeV)

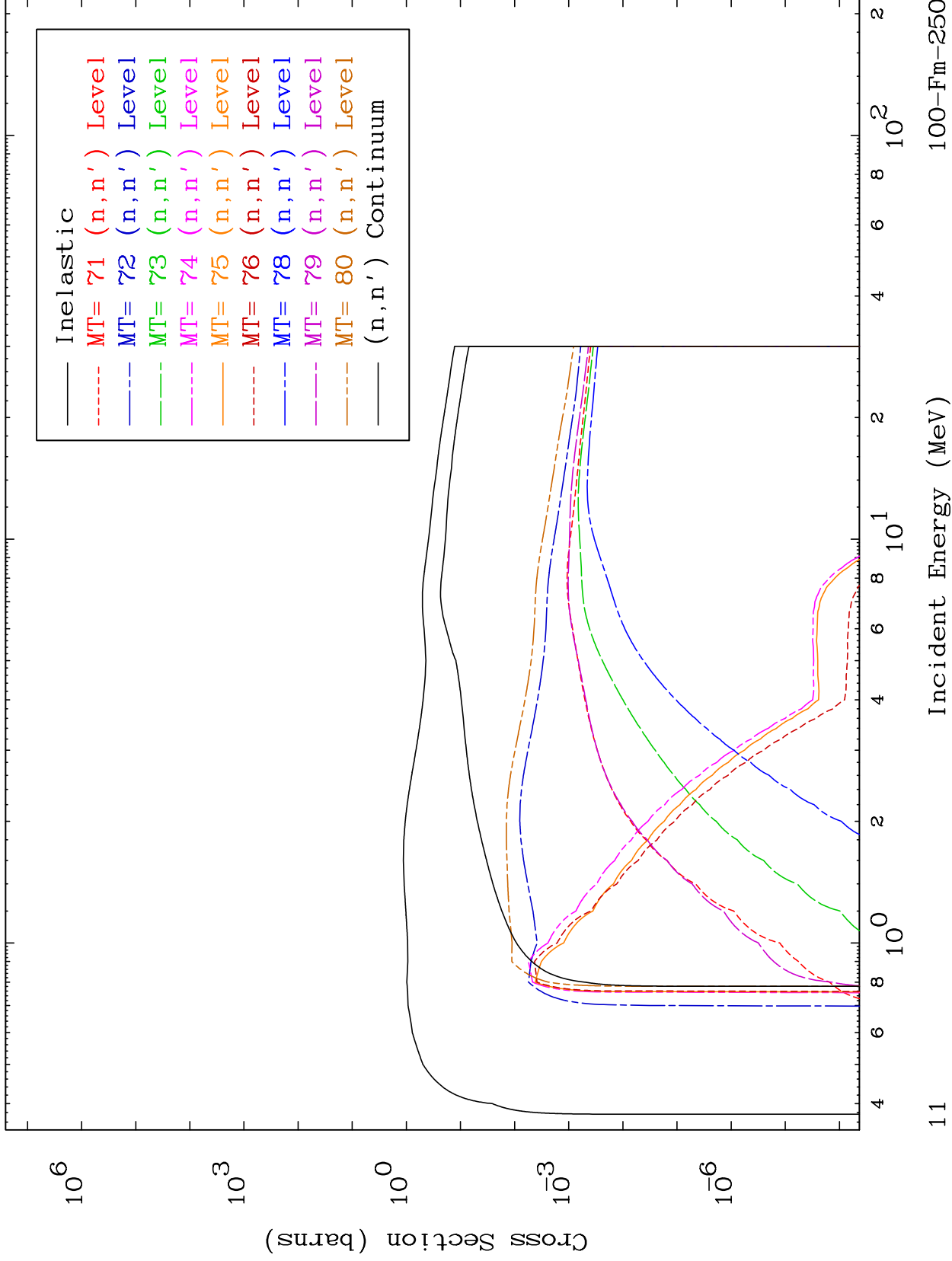
100-Fm-250

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(n,n') Levels

100-Fm-250

293 Kelvin Cross Sections



11

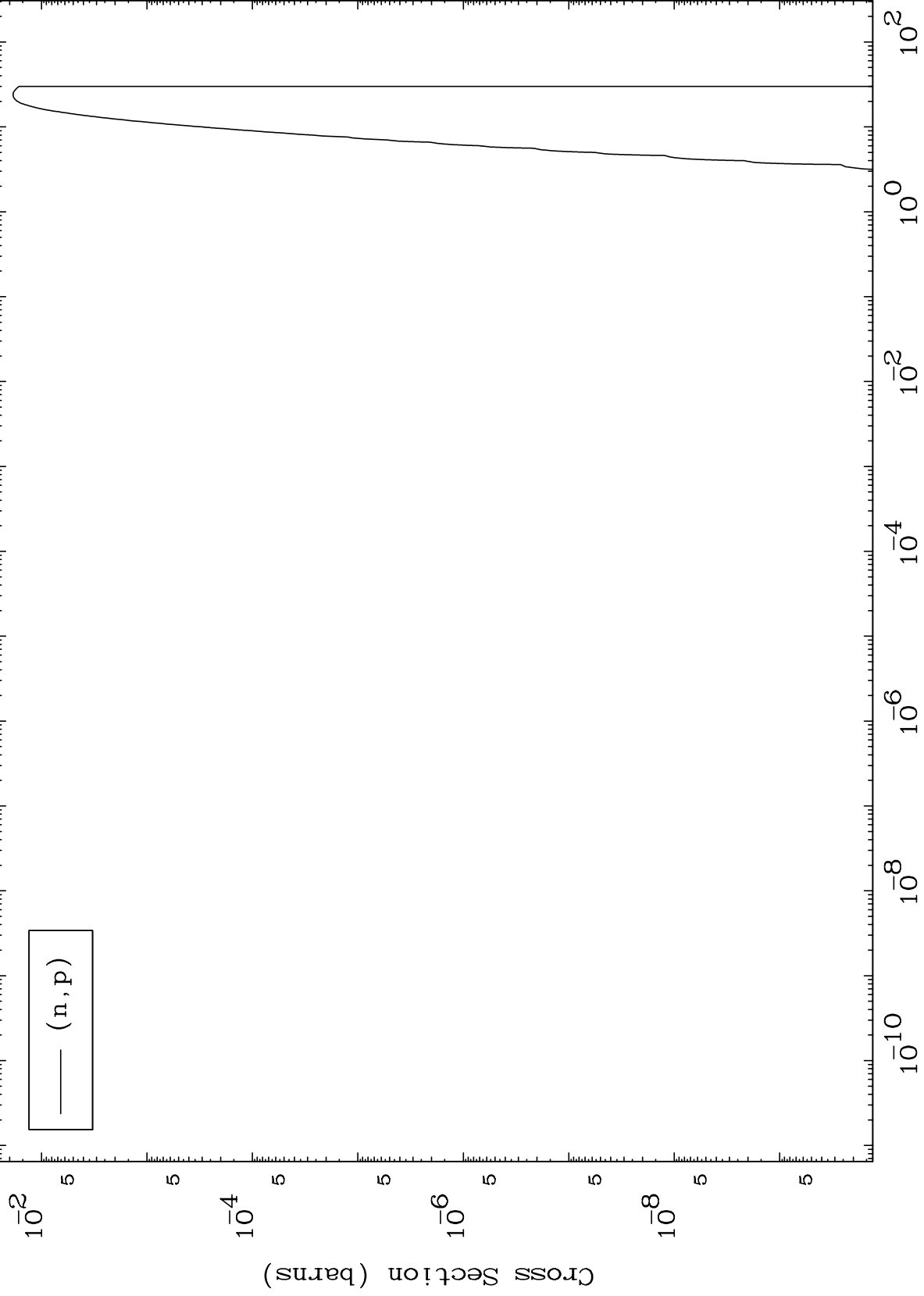
Incident Energy (MeV)

100-Fm-250

MAT 9931

(n,p) Levels
293 Kelvin Cross Sections

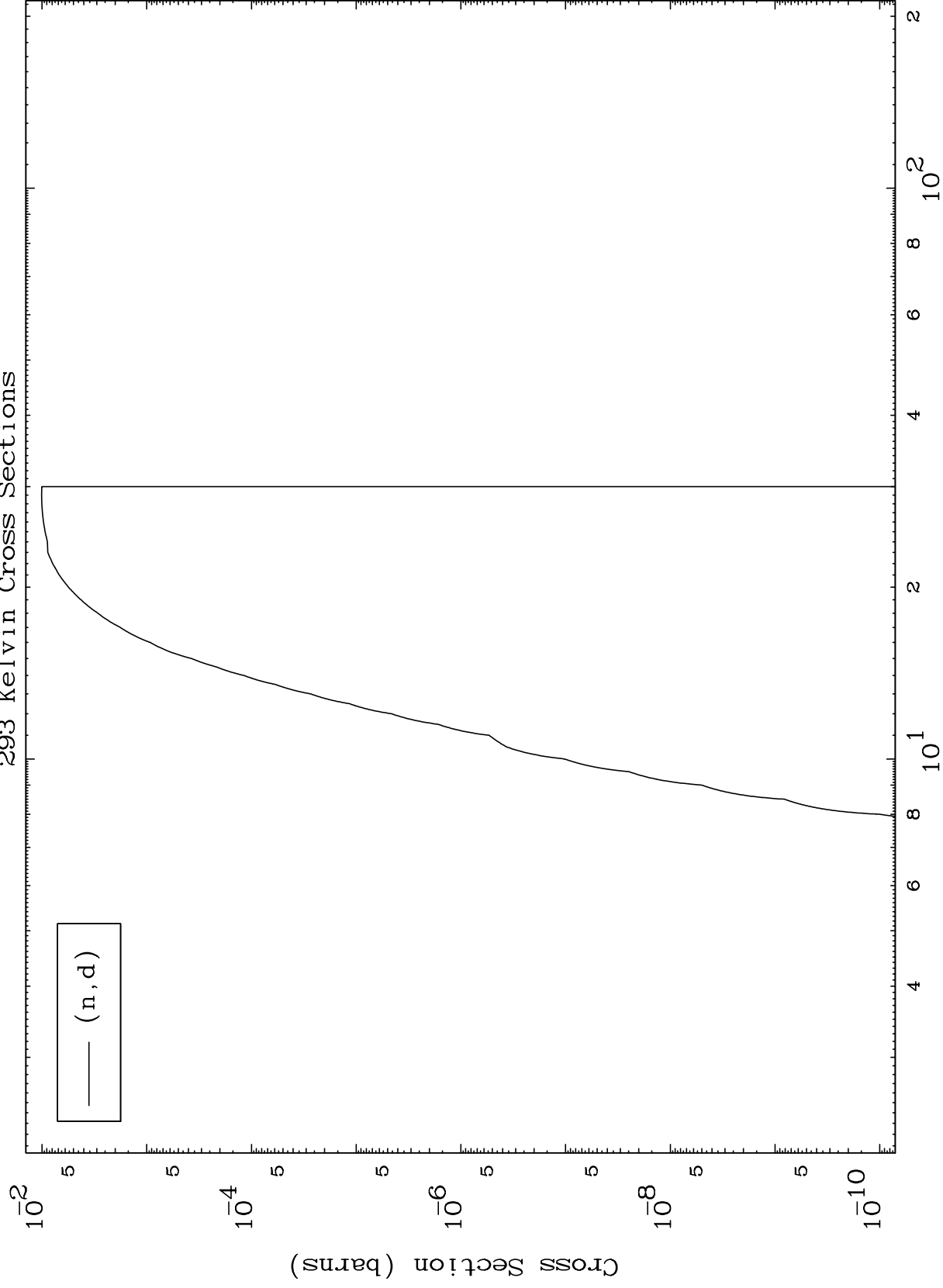
100-Fm-250



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(n,d) Levels
293 Kelvin Cross Sections

100-Fm-250



13

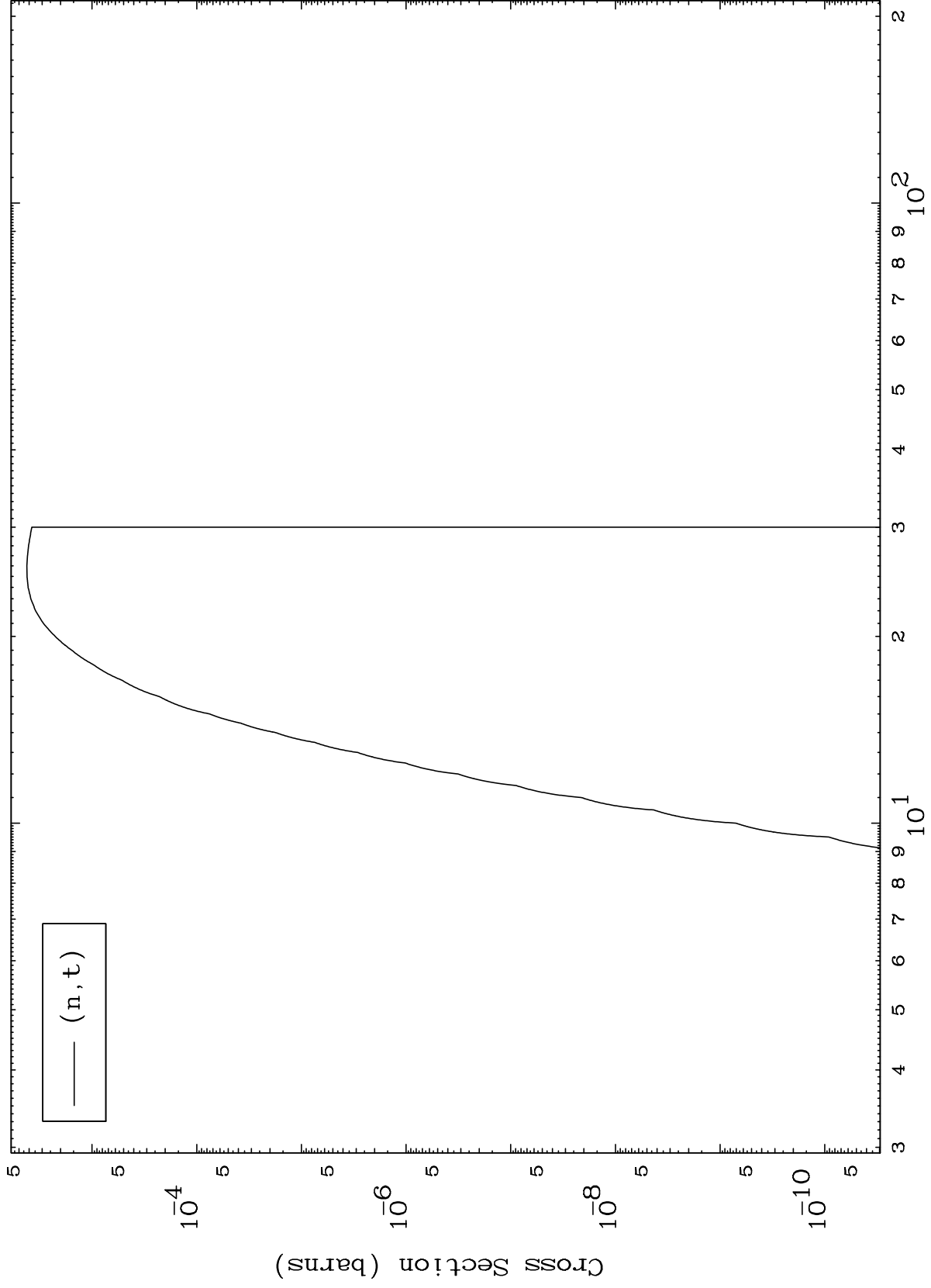
Incident Energy (MeV)

100-Fm-250

MAT 9931

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-250



14

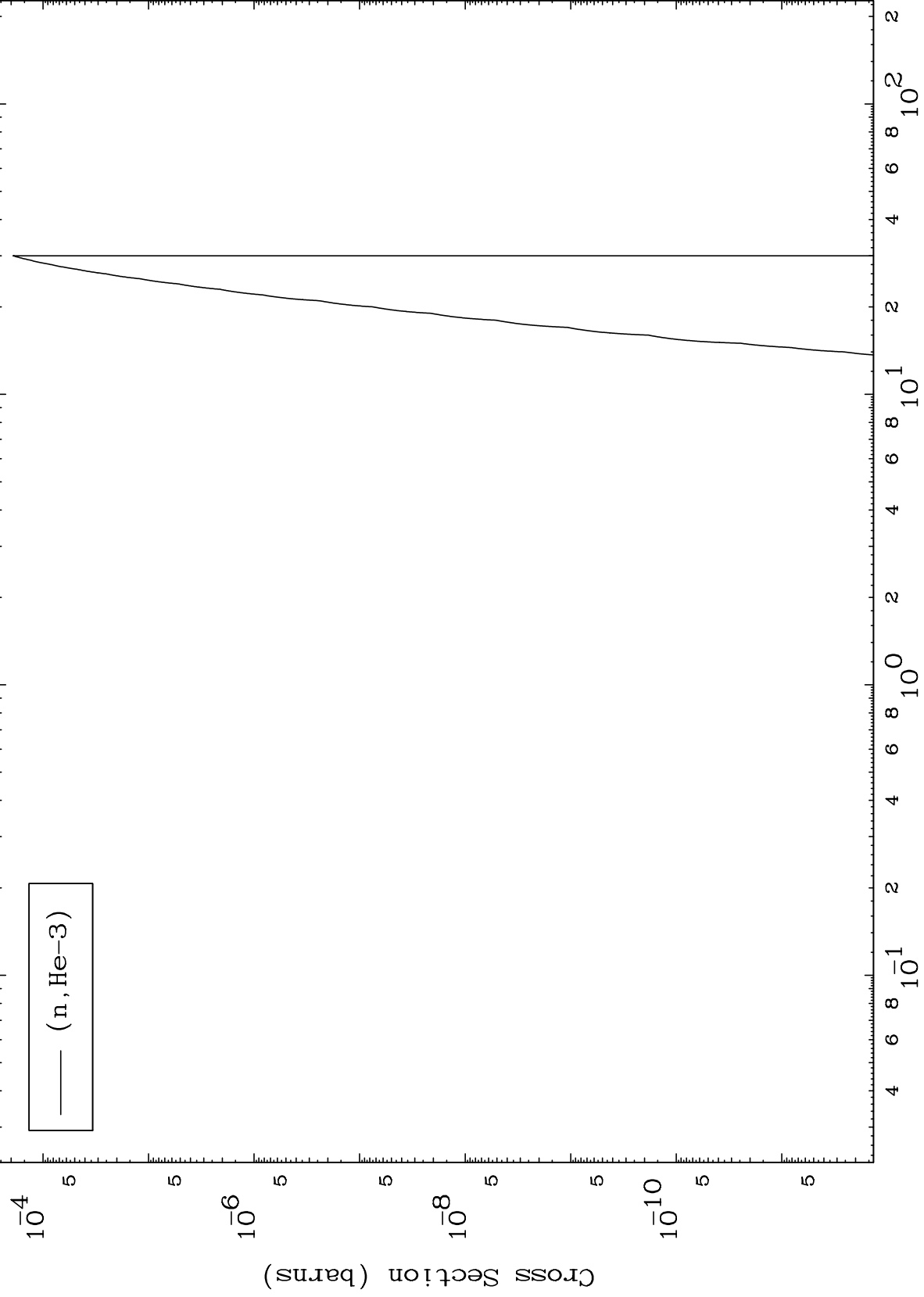
Incident Energy (MeV)

100-Fm-250

MAT 9931

(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-250



15

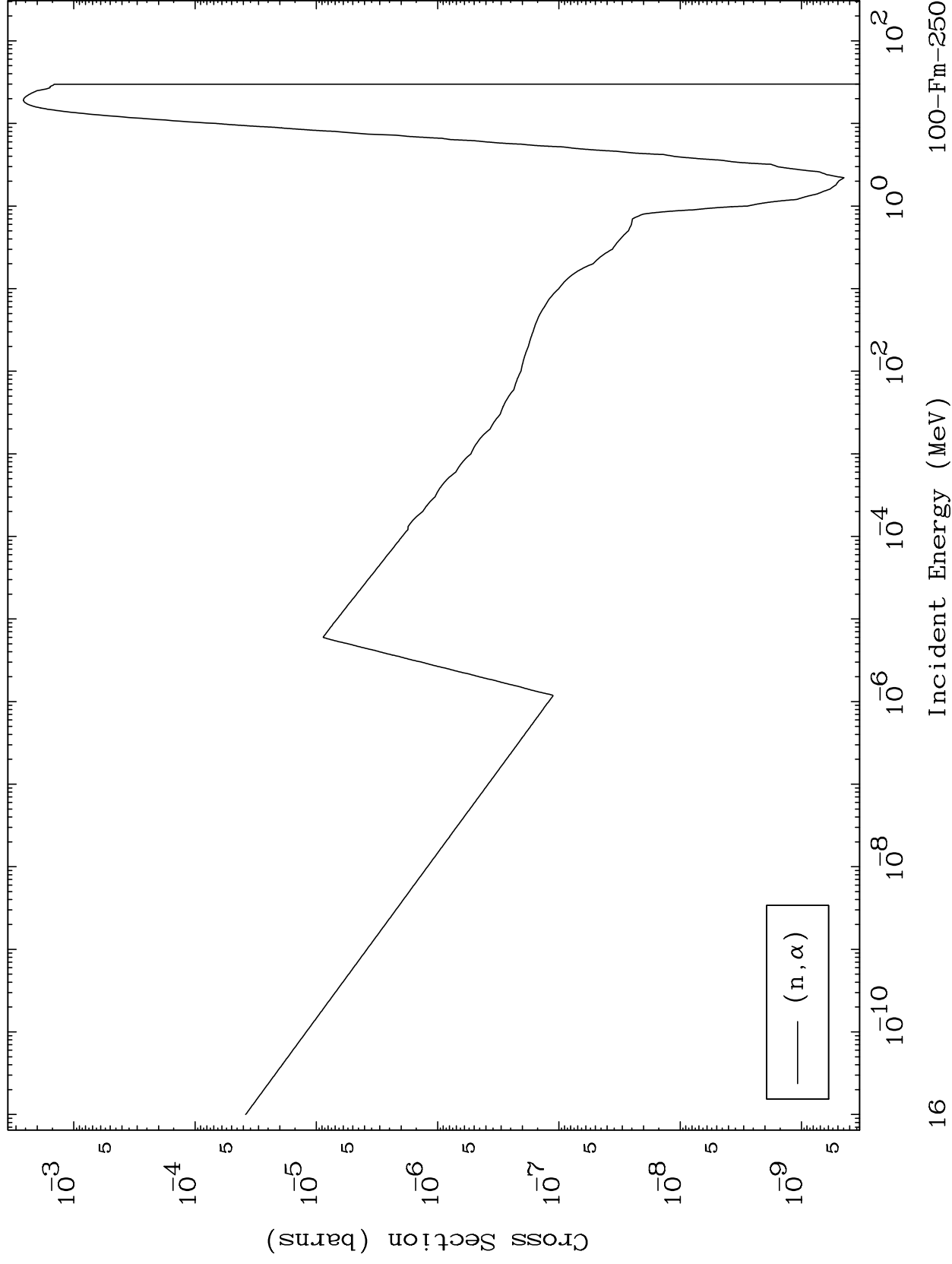
Incident Energy (MeV)

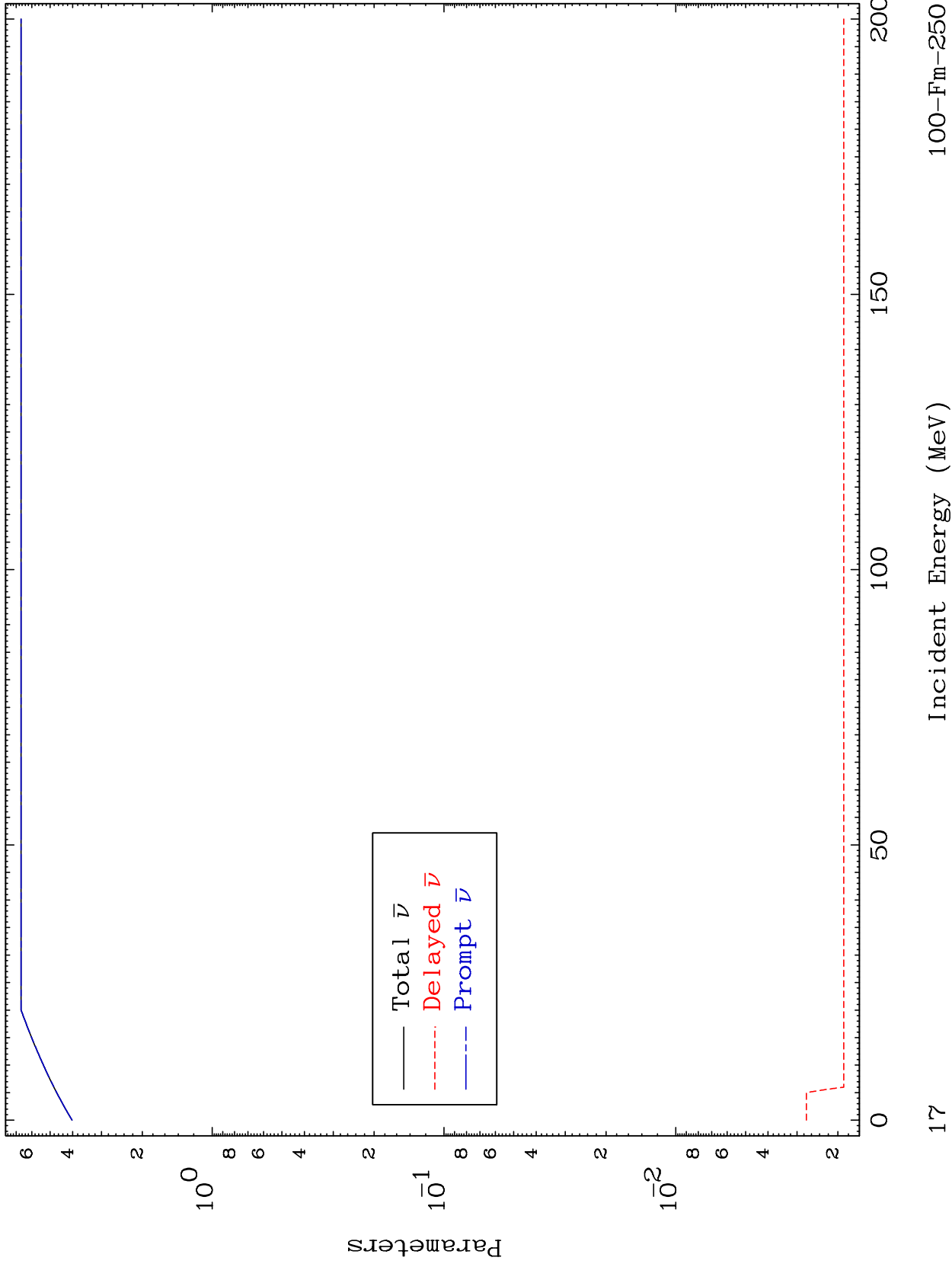
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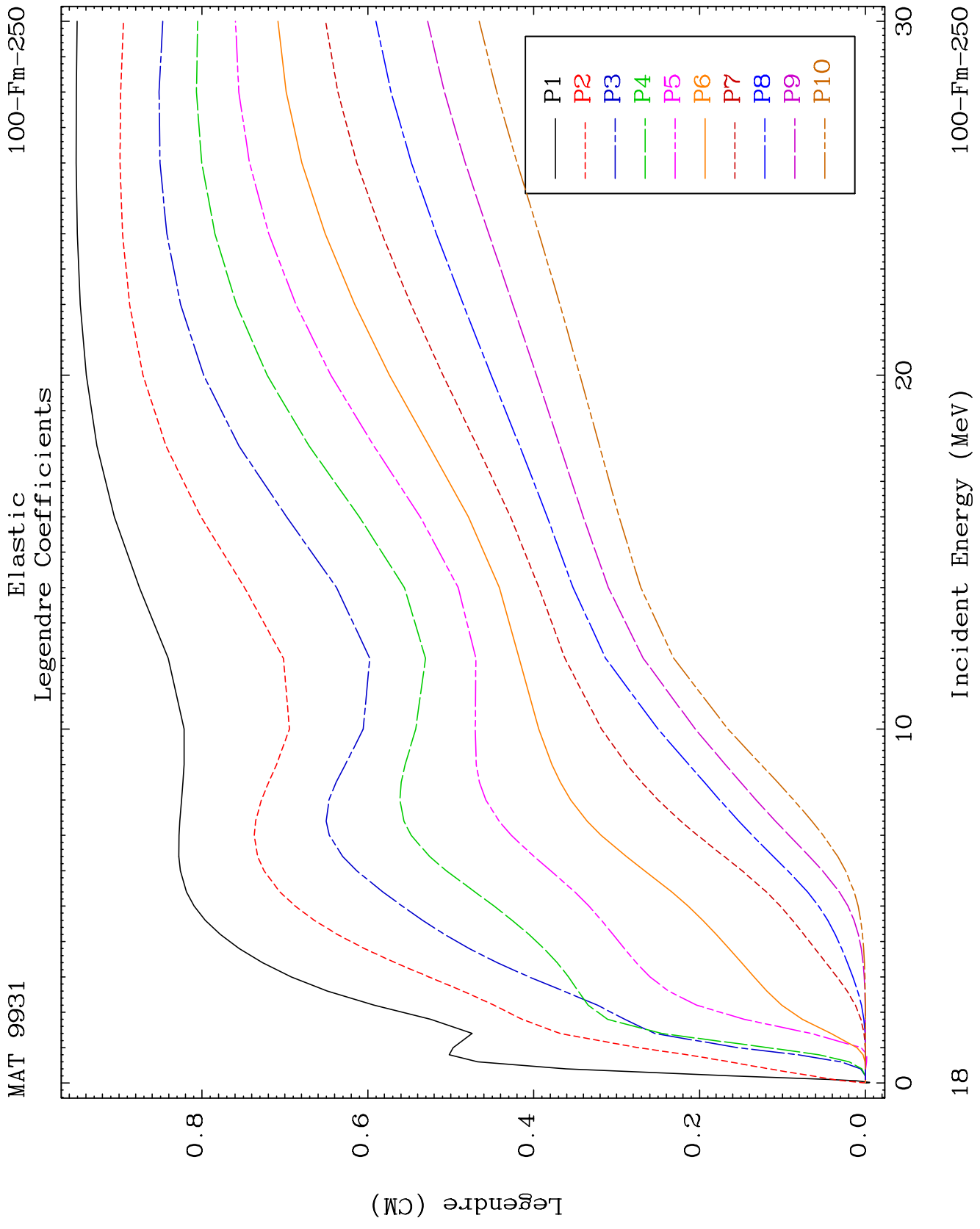
MAT 9931

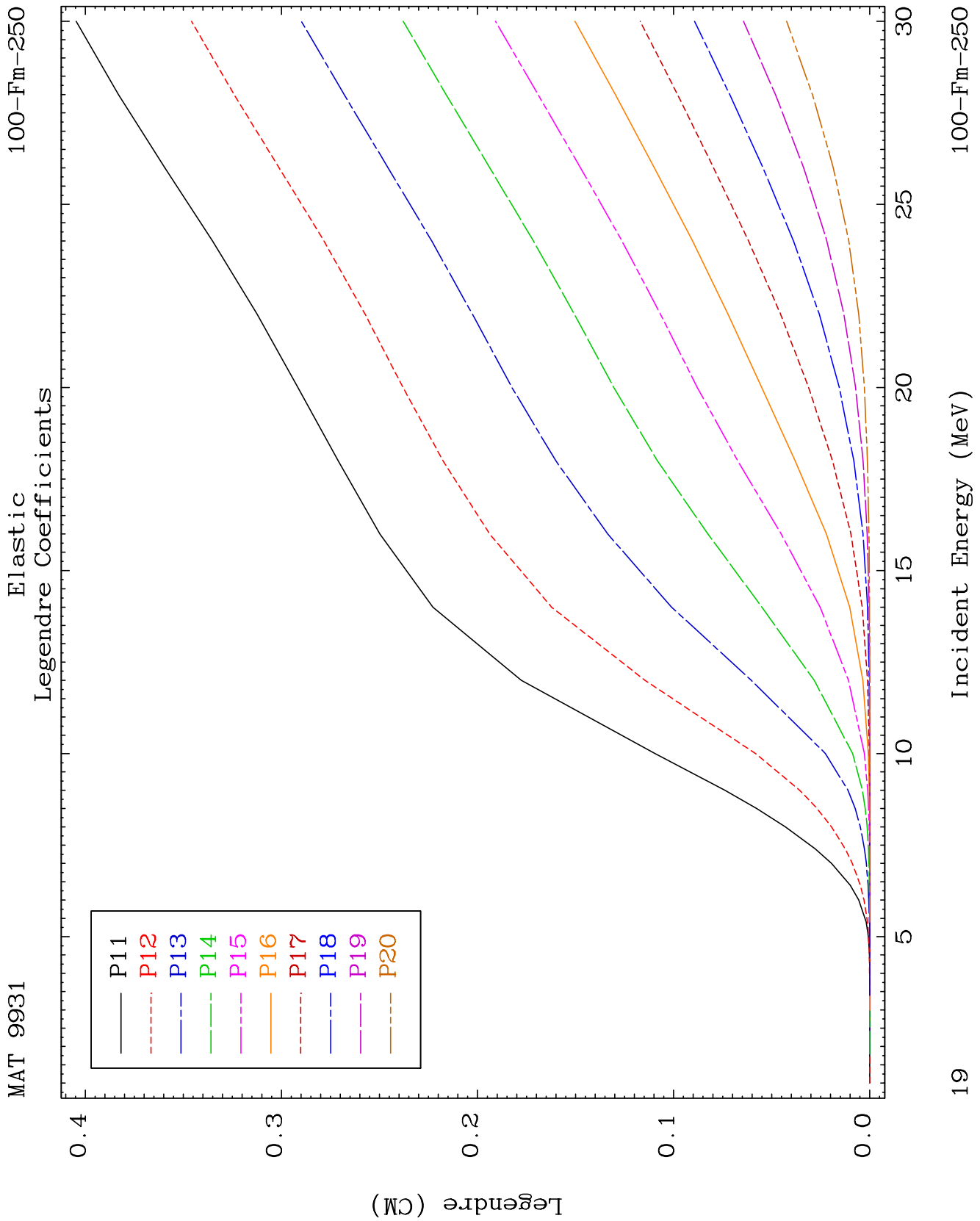
(n,α) Levels
293 Kelvin Cross Sections

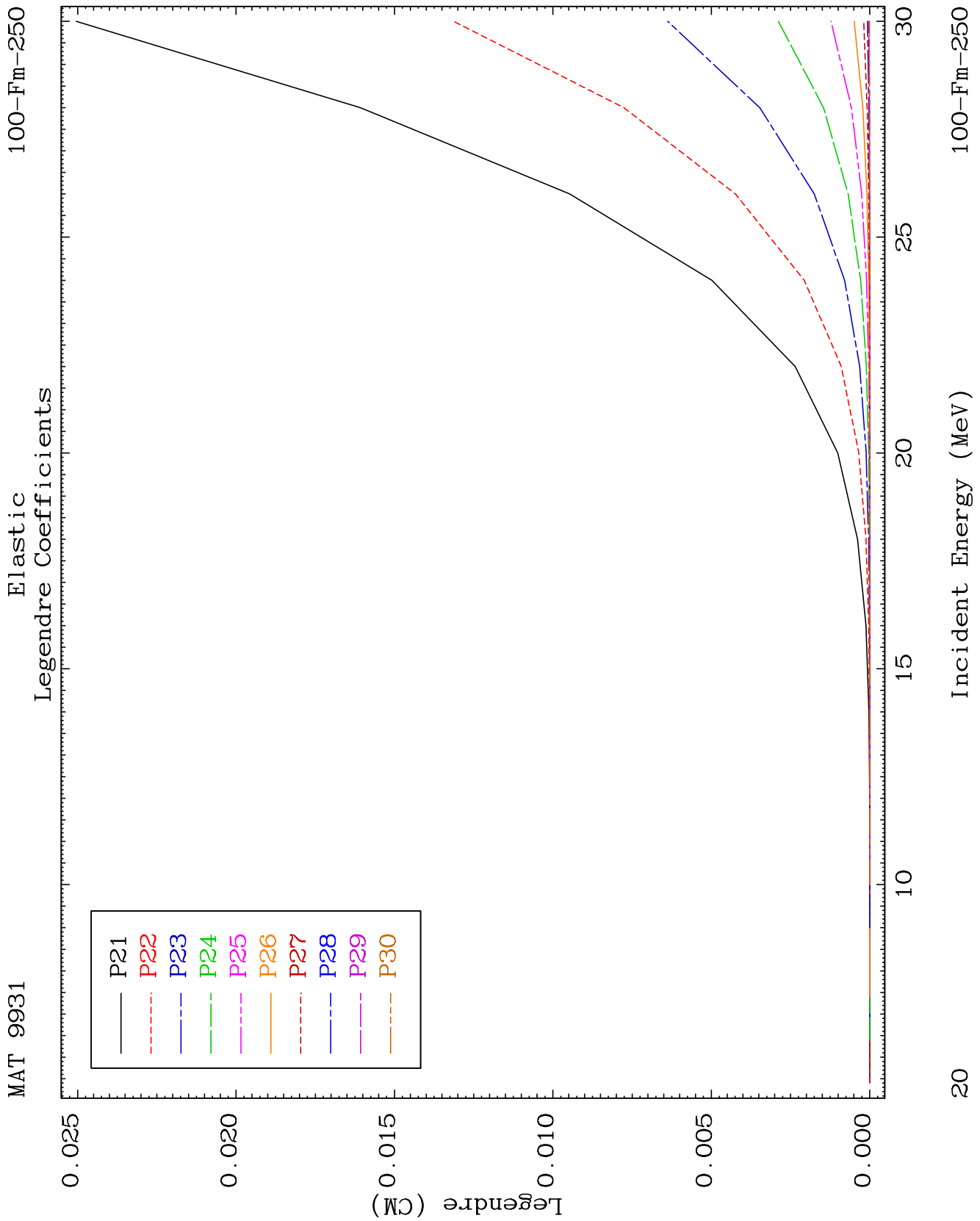
100-Fm-250

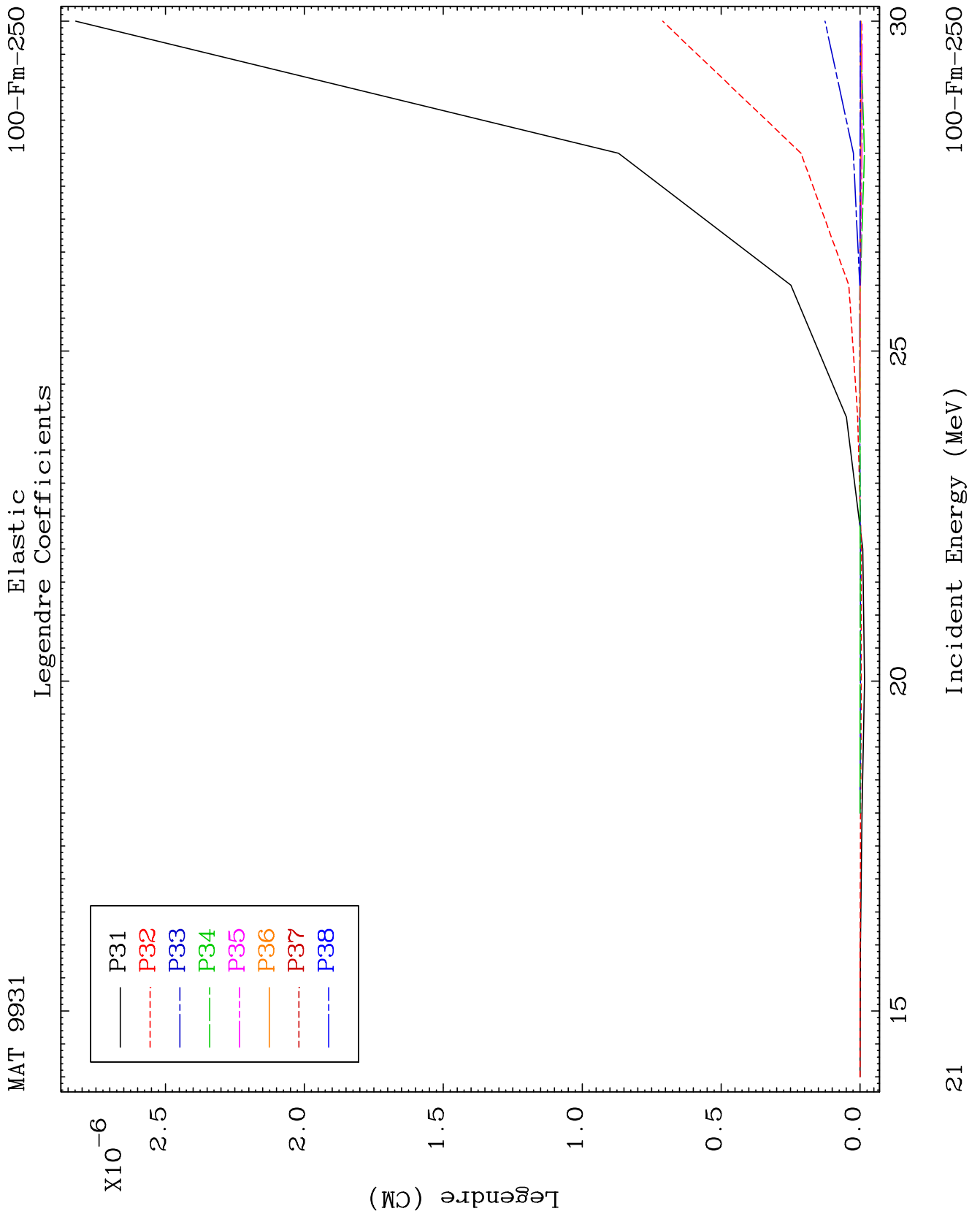








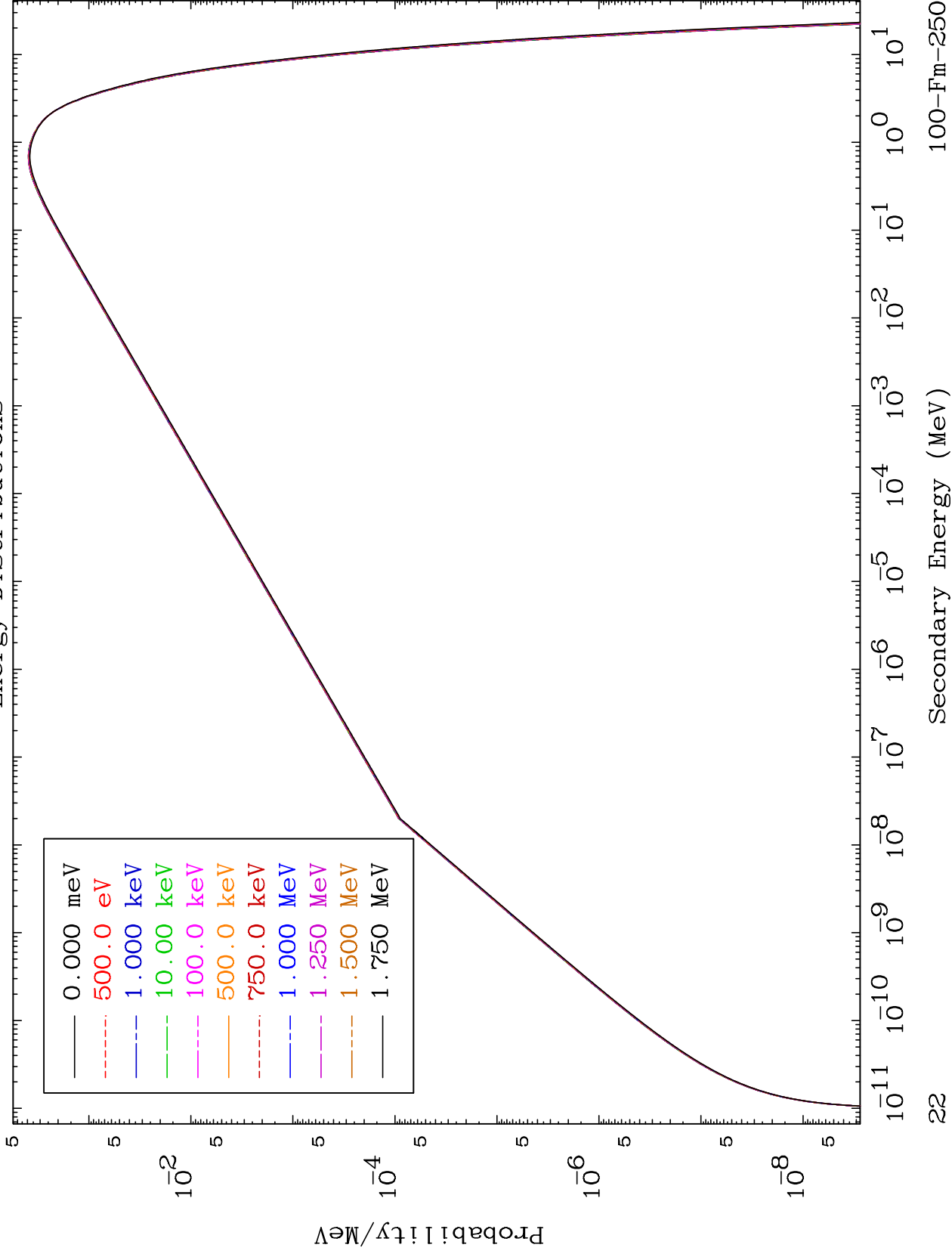




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Fission Energy Distributions

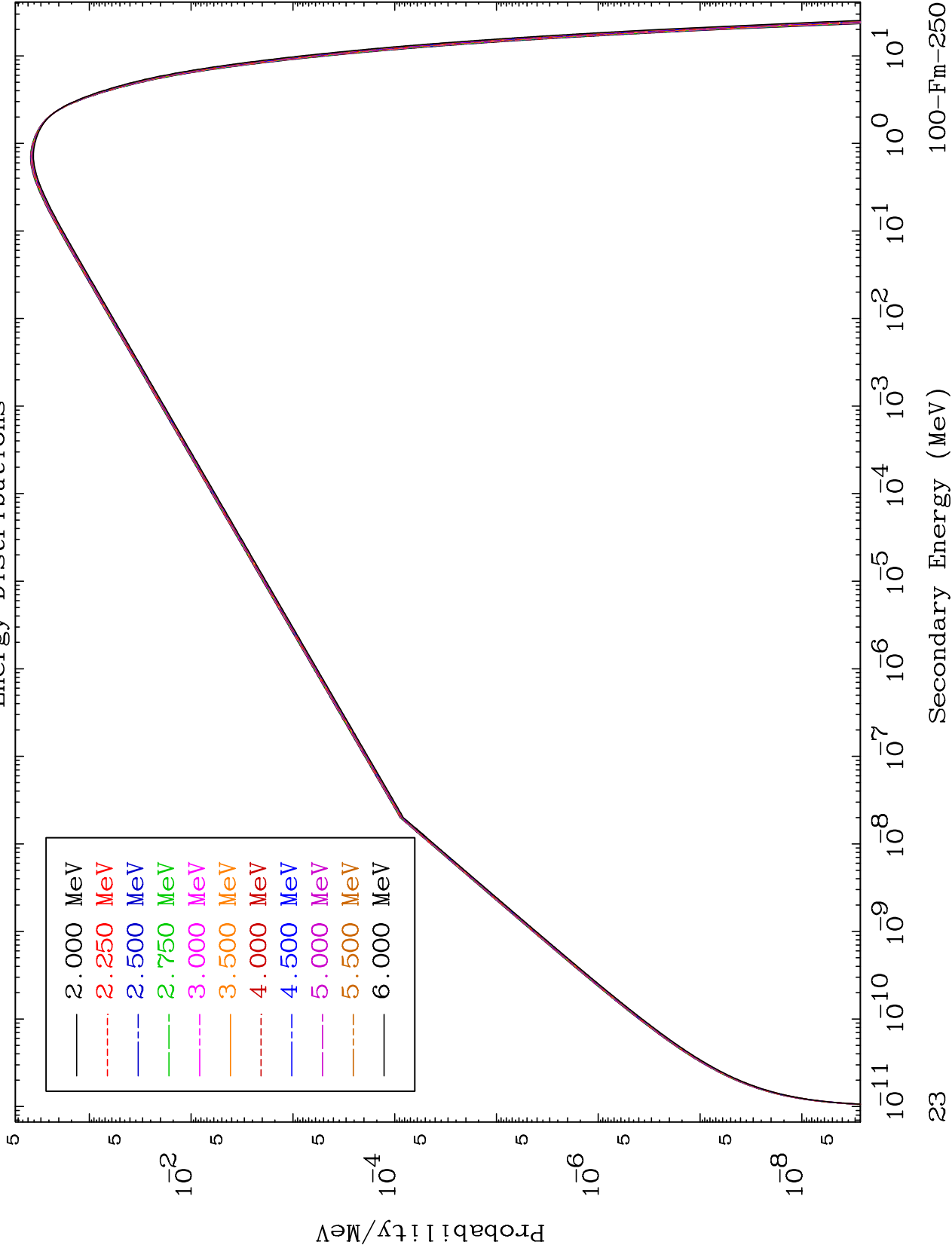
100-Fm-250



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Fission Energy Distributions

100-Fm-250



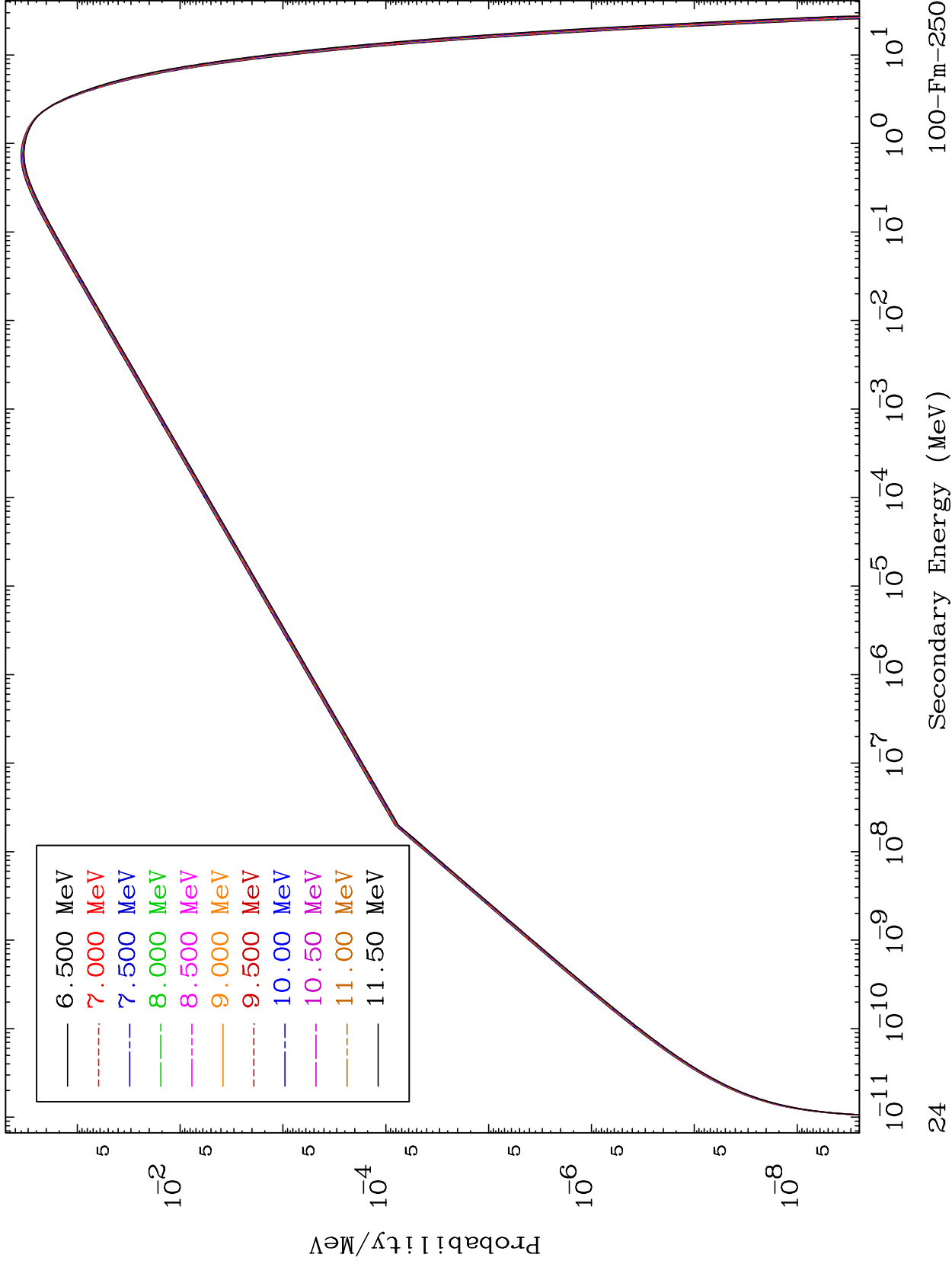
100-Fm-250

Secondary Energy (MeV)

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Fission Energy Distributions

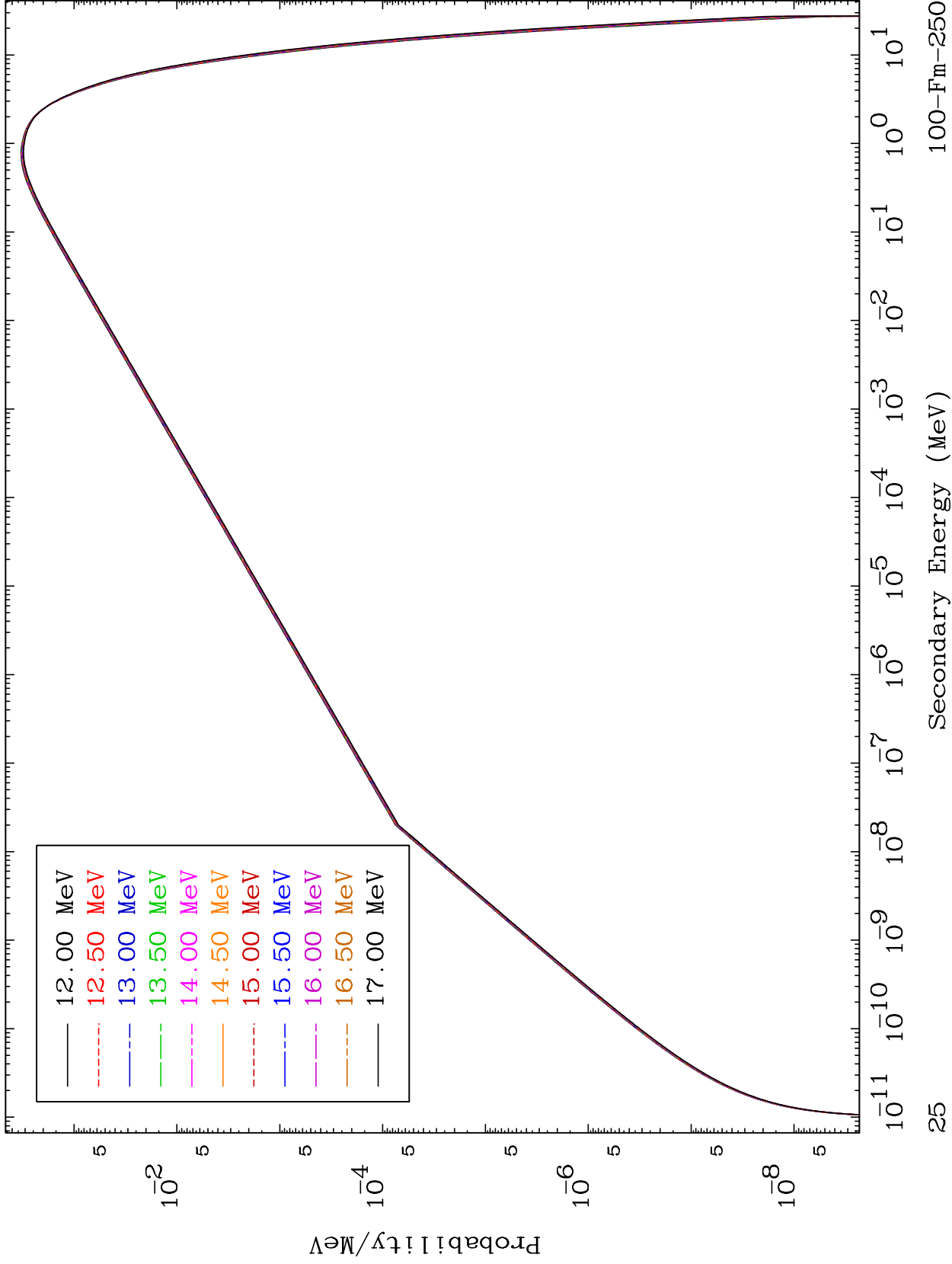
100-Fm-250



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Fission Energy Distributions

100-Fm-250



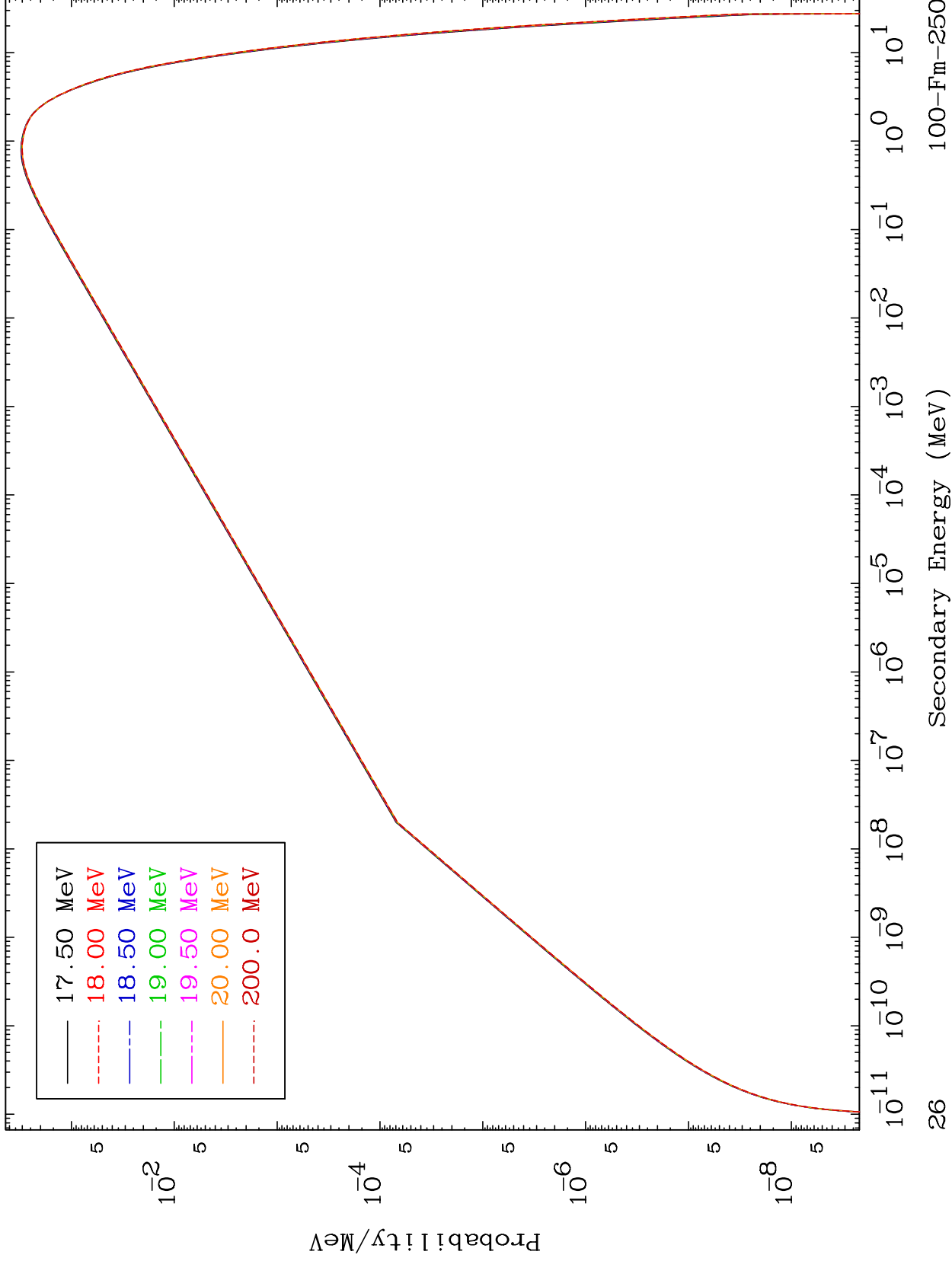
25

100-Fm-250

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Fission Energy Distributions

100-Fm-250

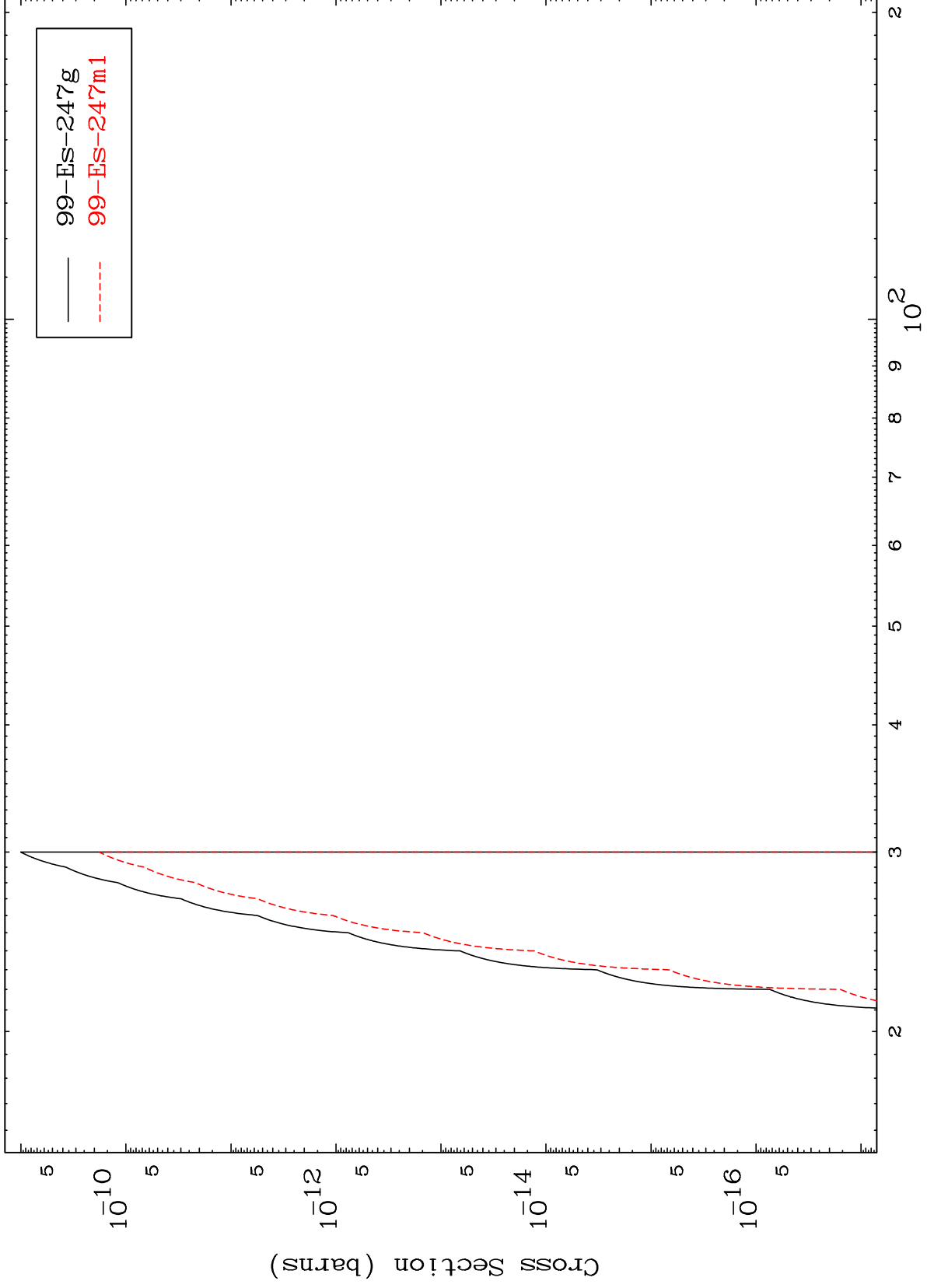


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(n,2n) d

100-Fm-250

Radionuclide Production Cross Section



27

Incident Energy (MeV)

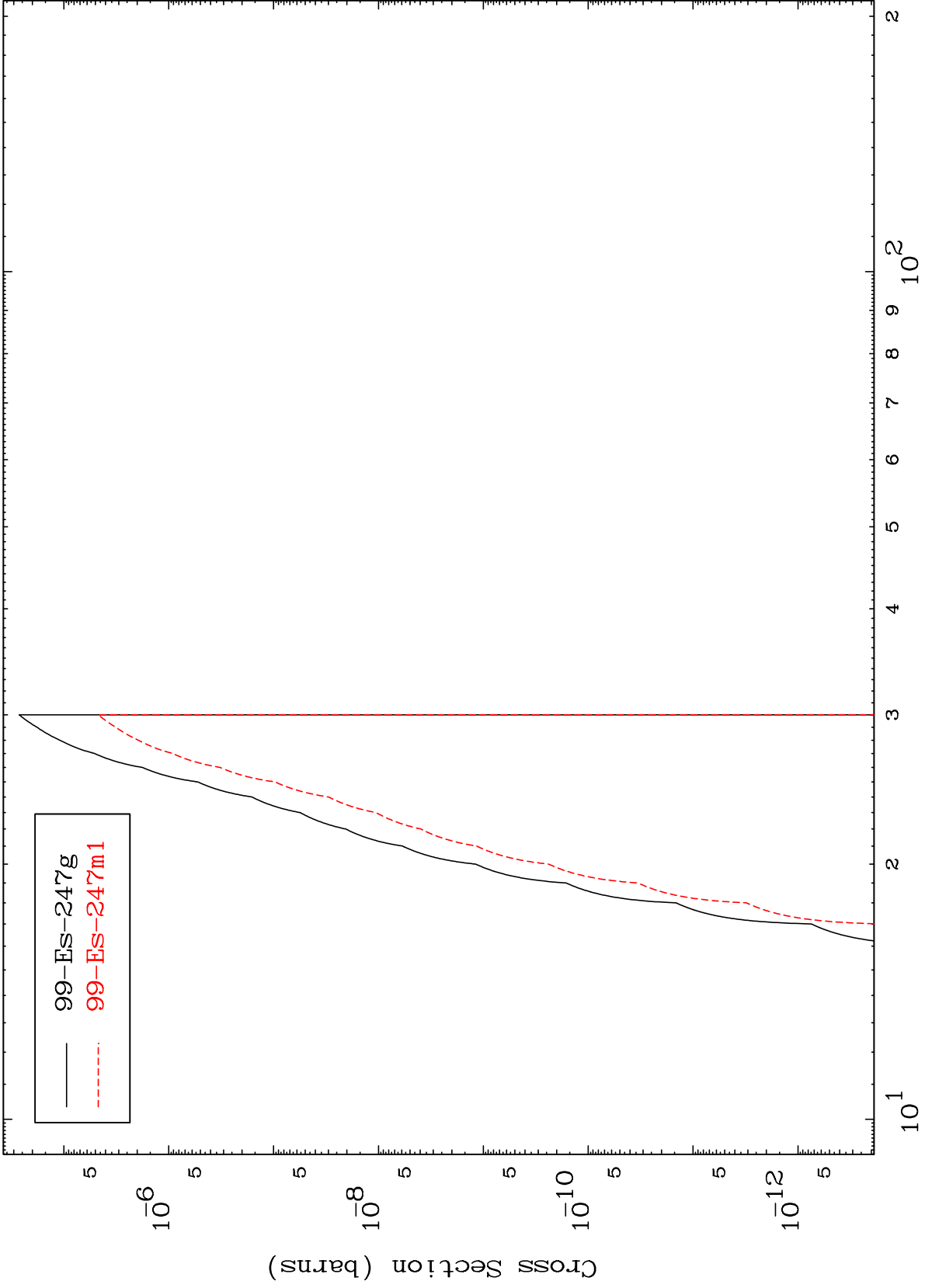
100-Fm-250

MAT 9931

(n,n') t

100-Fm-250

Radionuclide Production Cross Section



28

Incident Energy (MeV)

10²

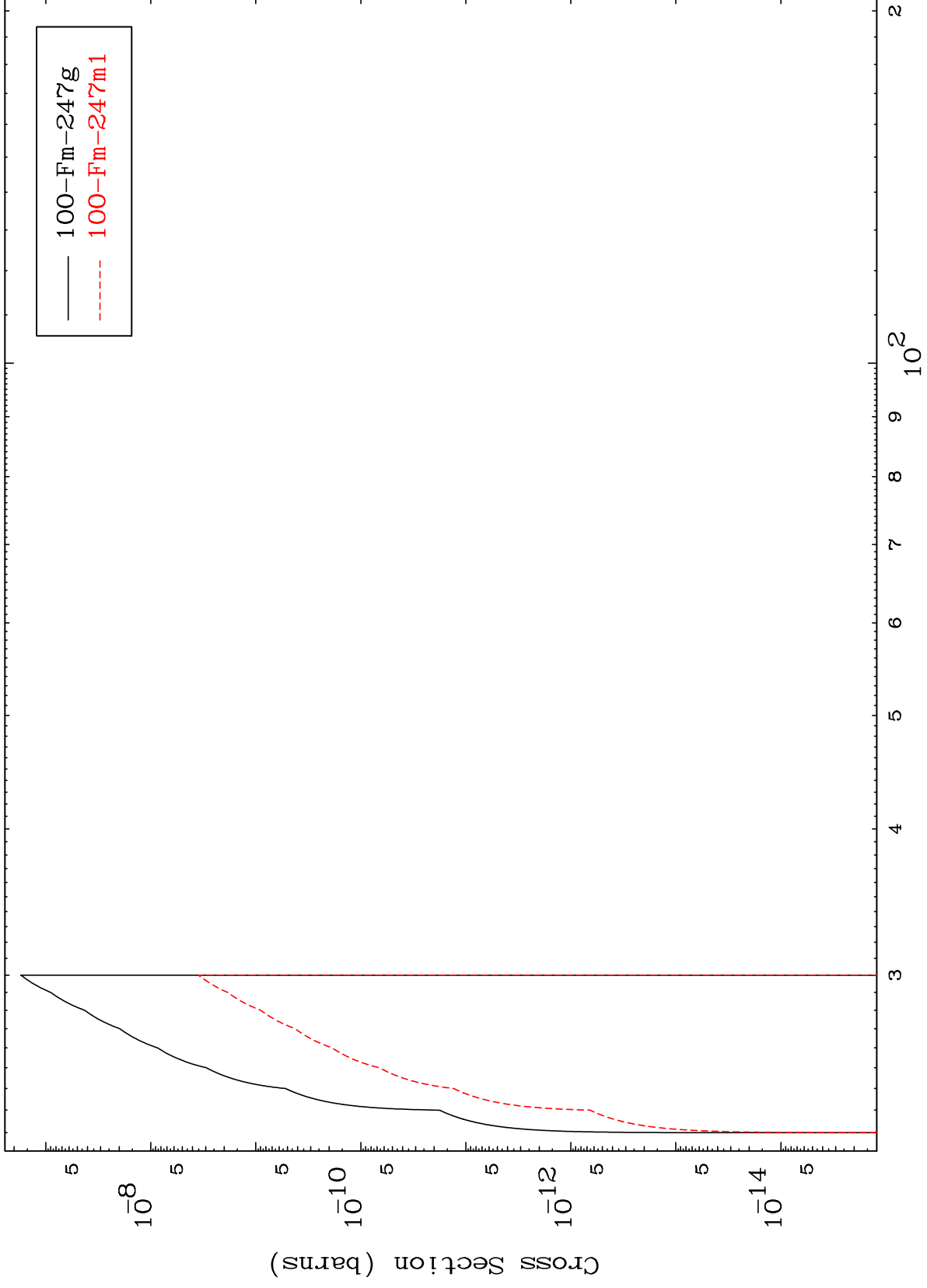
100-Fm-250

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(n,4n)

100-Fm-250

Radionuclide Production Cross Section



29

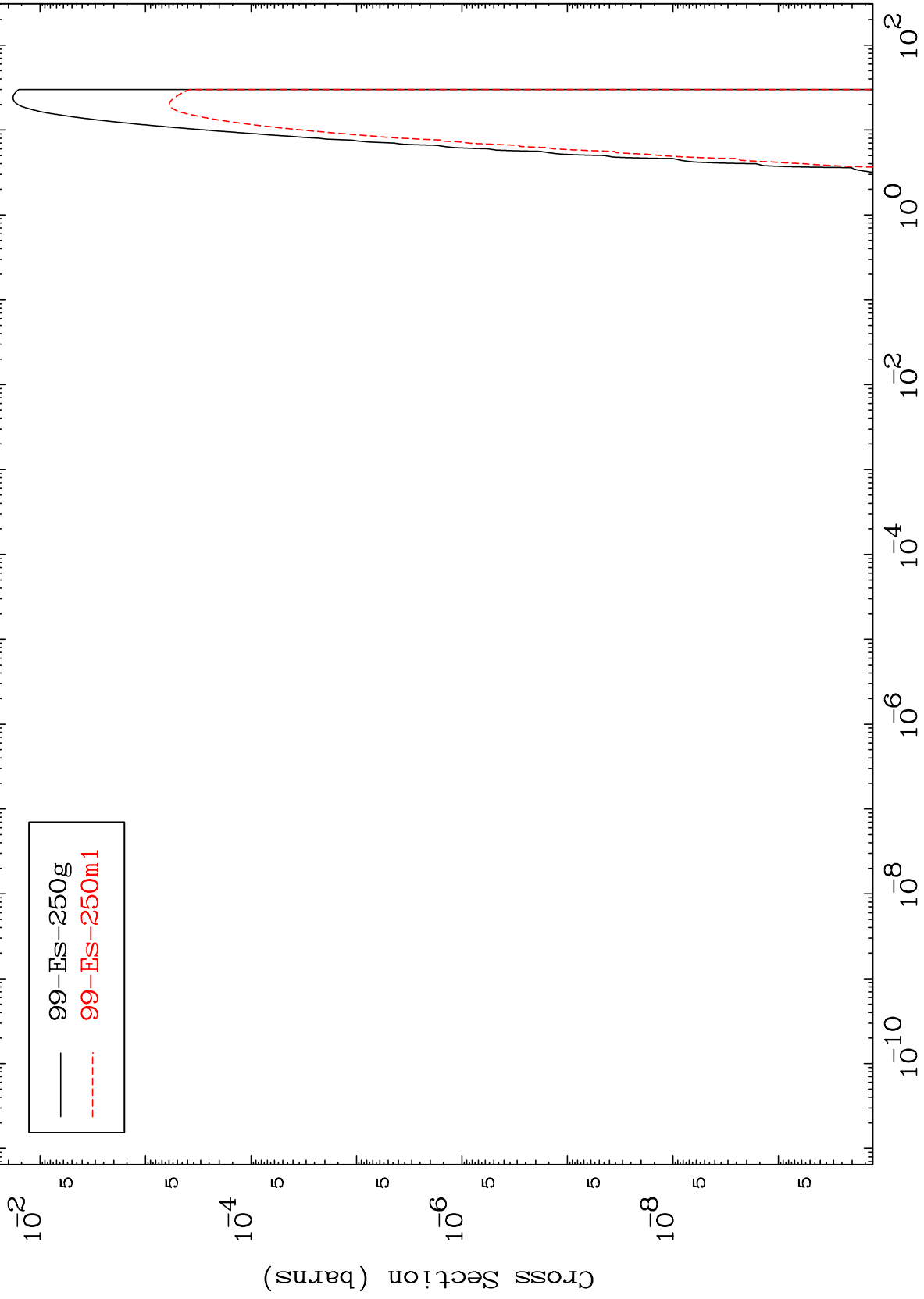
Incident Energy (MeV)

100-Fm-250

MAT 9931

100-Fm-250

(n,p)
Radionuclide Production Cross Section



— 99-Es-250g
- - - 99-Es-250m1

100-Fm-250

Incident Energy (MeV)