

Program EVALPLOT
(Version 2021-1)

by

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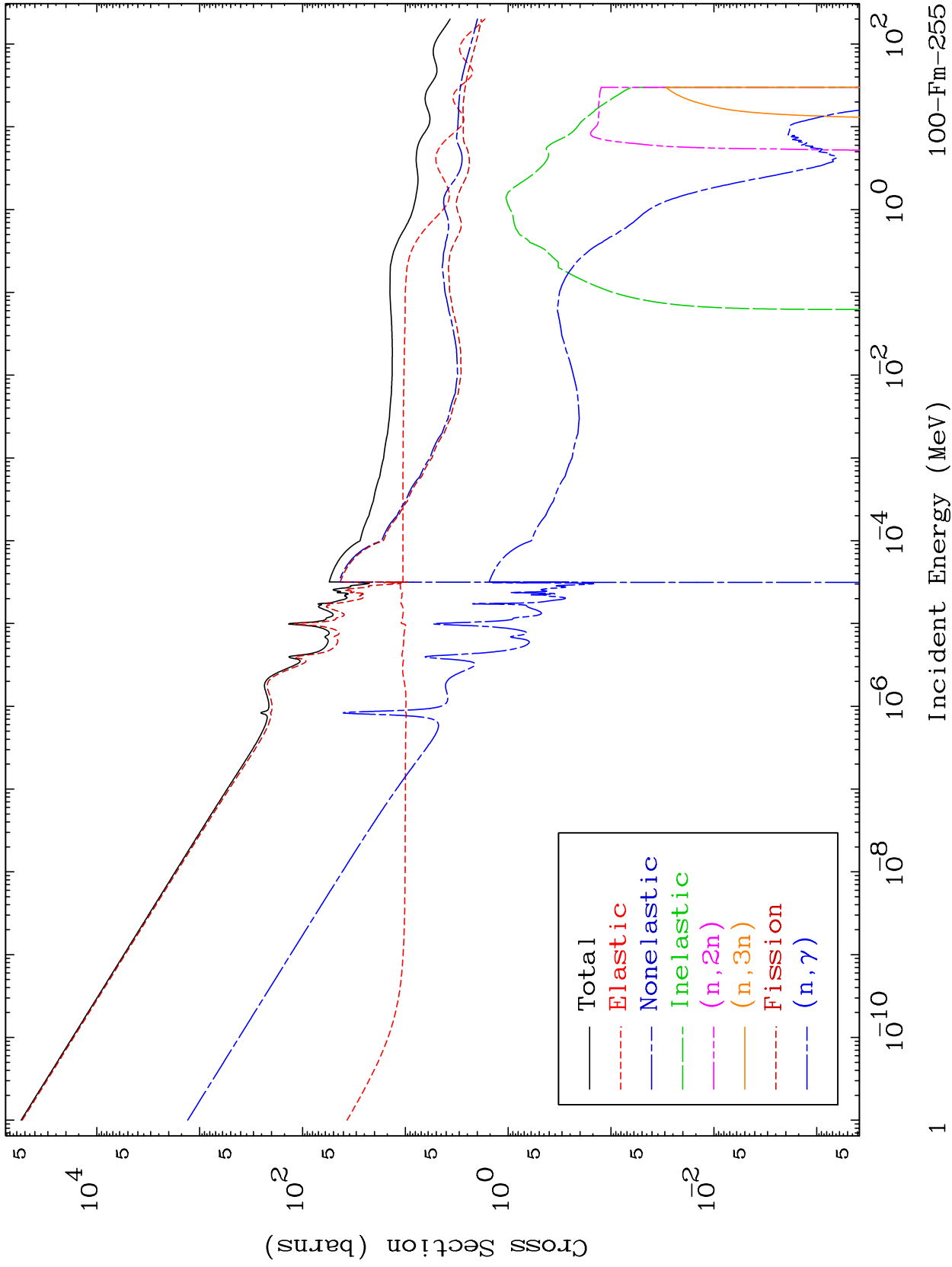
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9936

Neutron Major
293 Kelvin Cross Sections

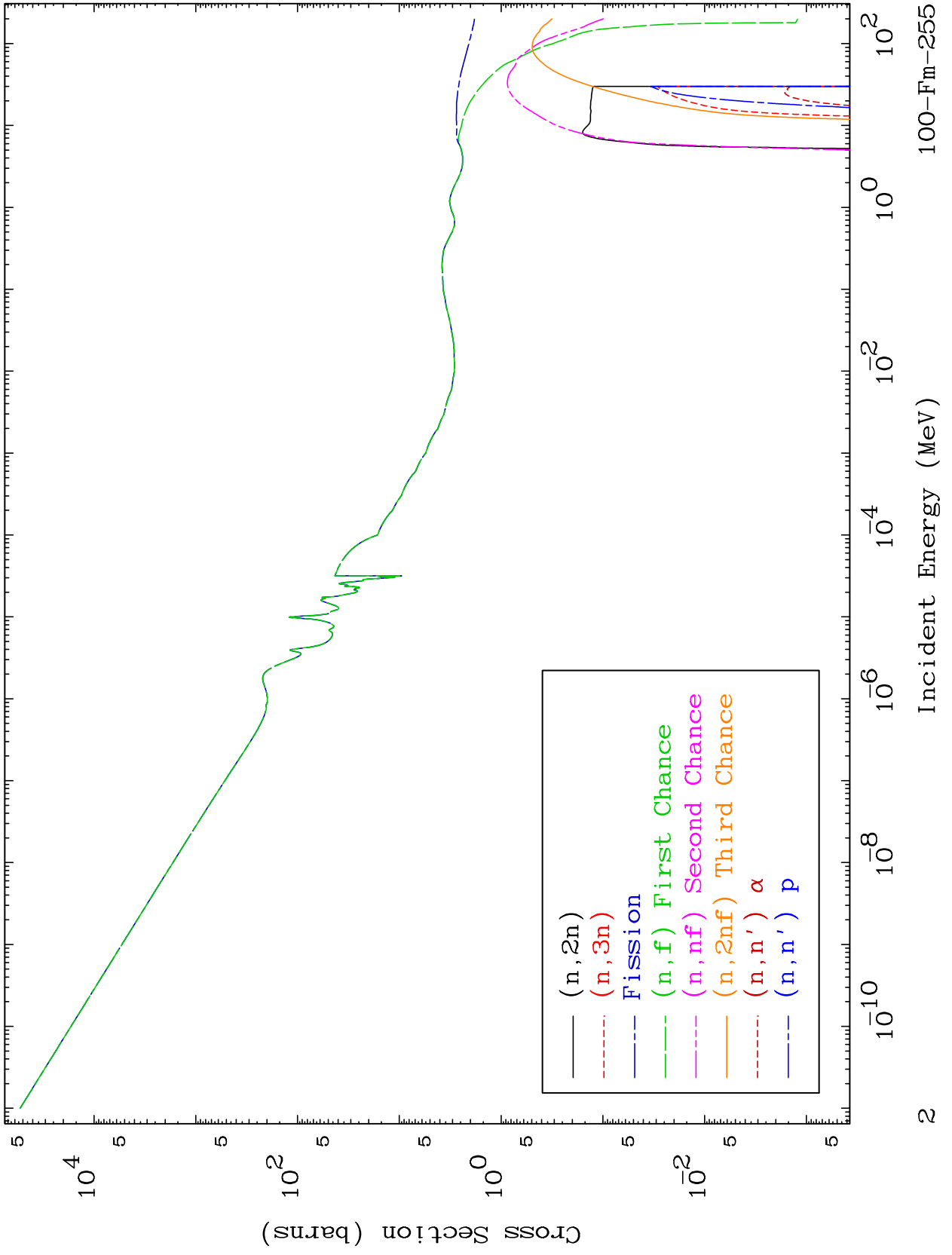
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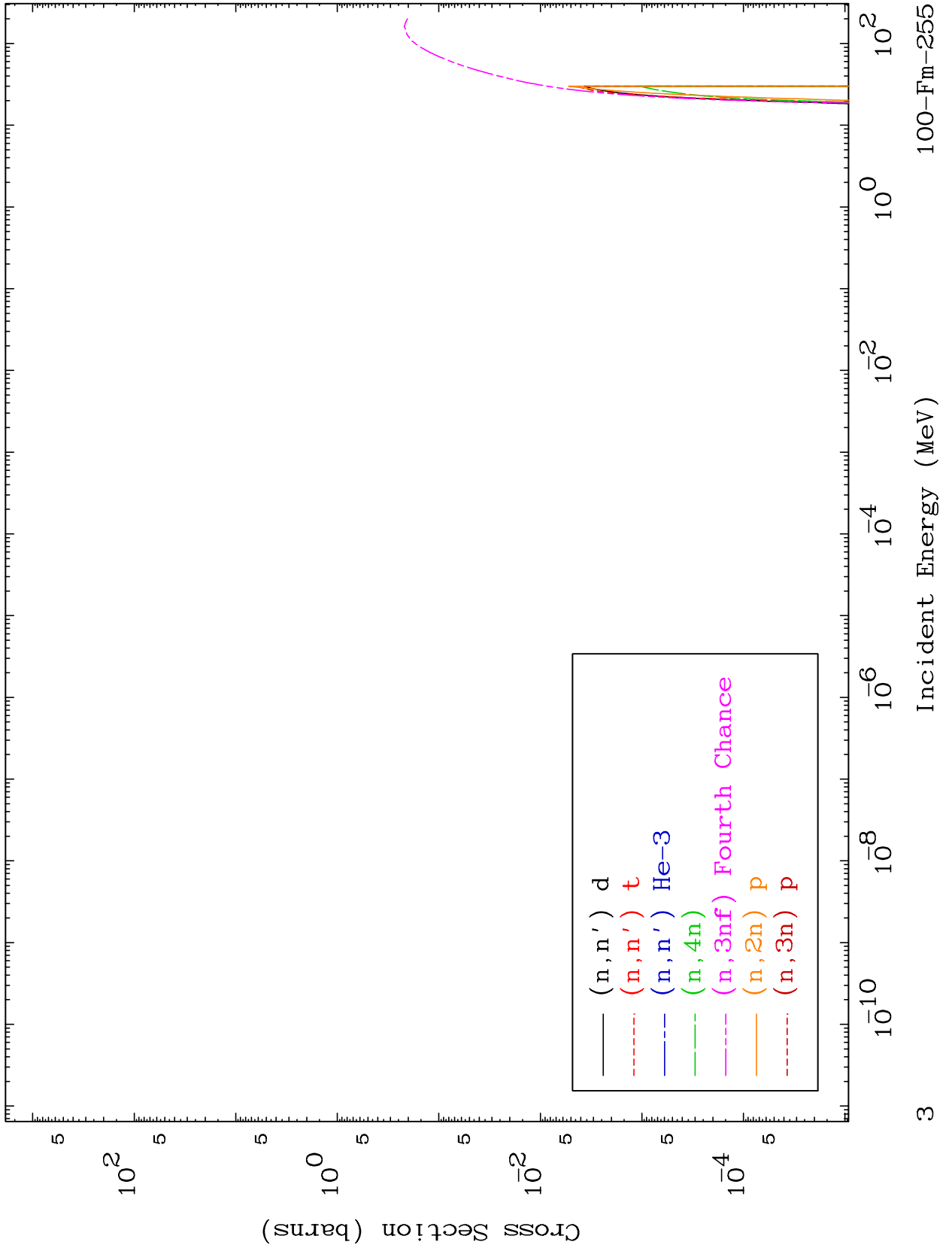


MAT 9936

Neutron Absorption
293 Kelvin Cross Sections

100-Fm-255

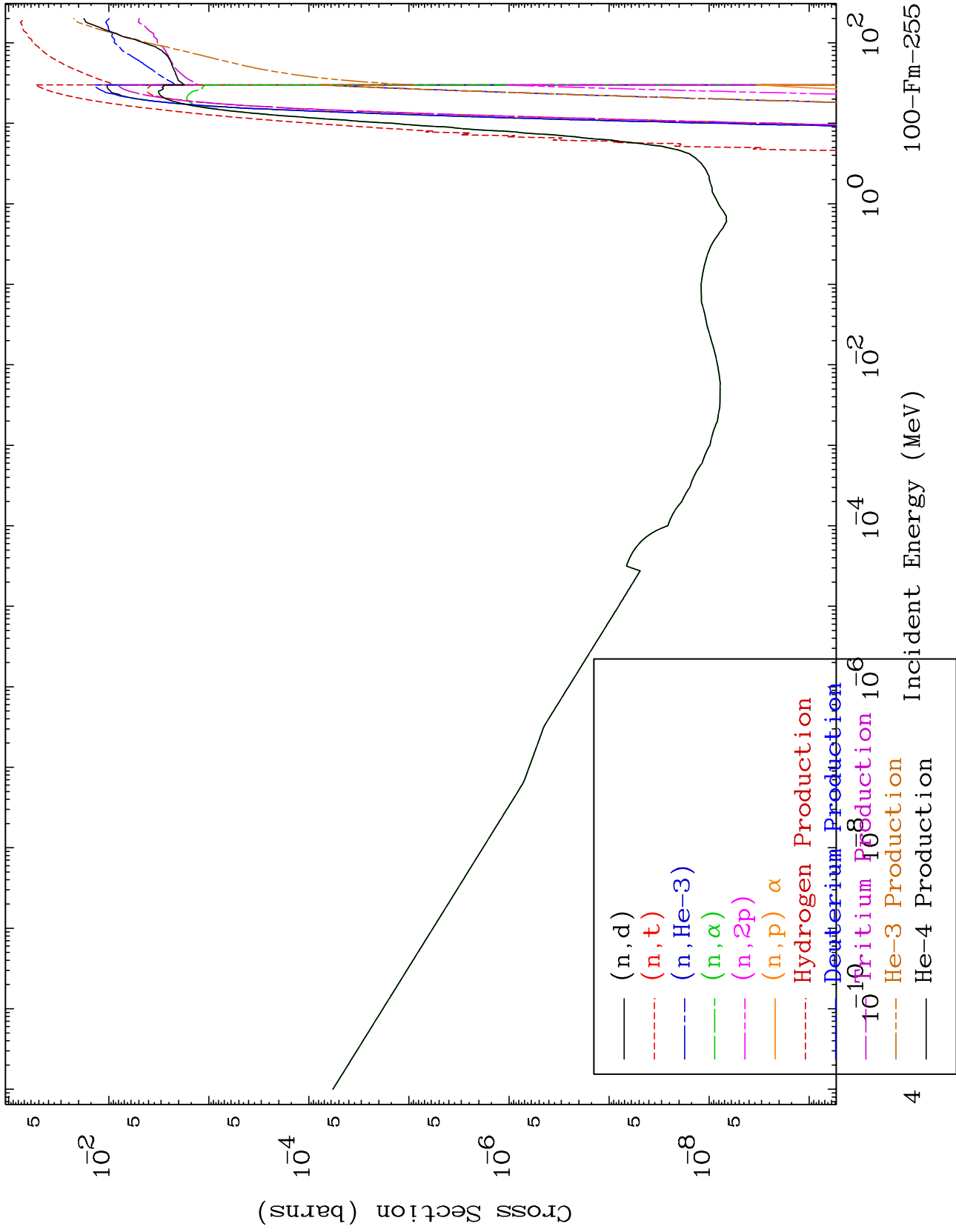




MAT 9936

Neutron Absorption
293 Kelvin Cross Sections

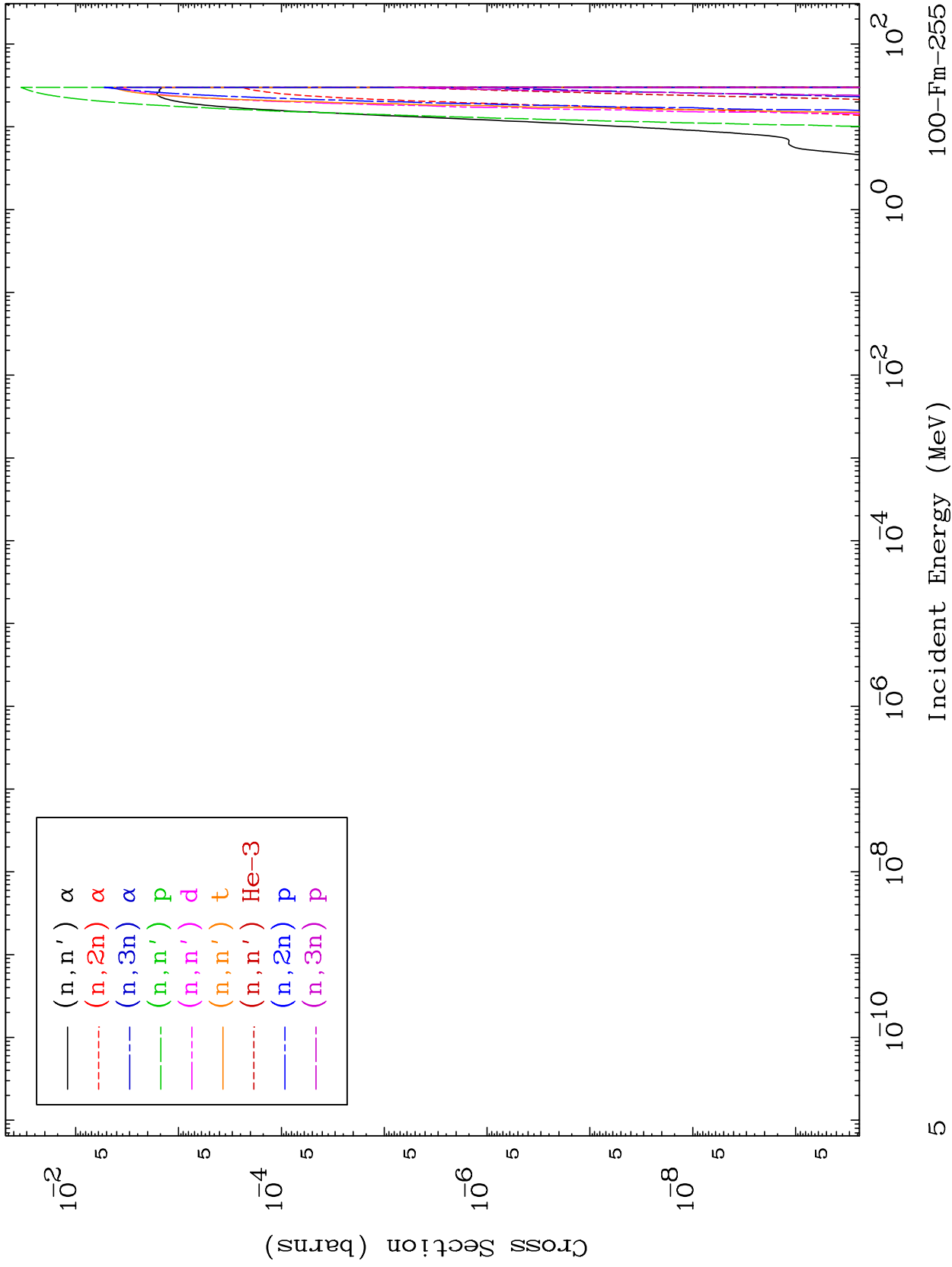
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MAT 9936

Charged Particle
293 Kelvin Cross Sections

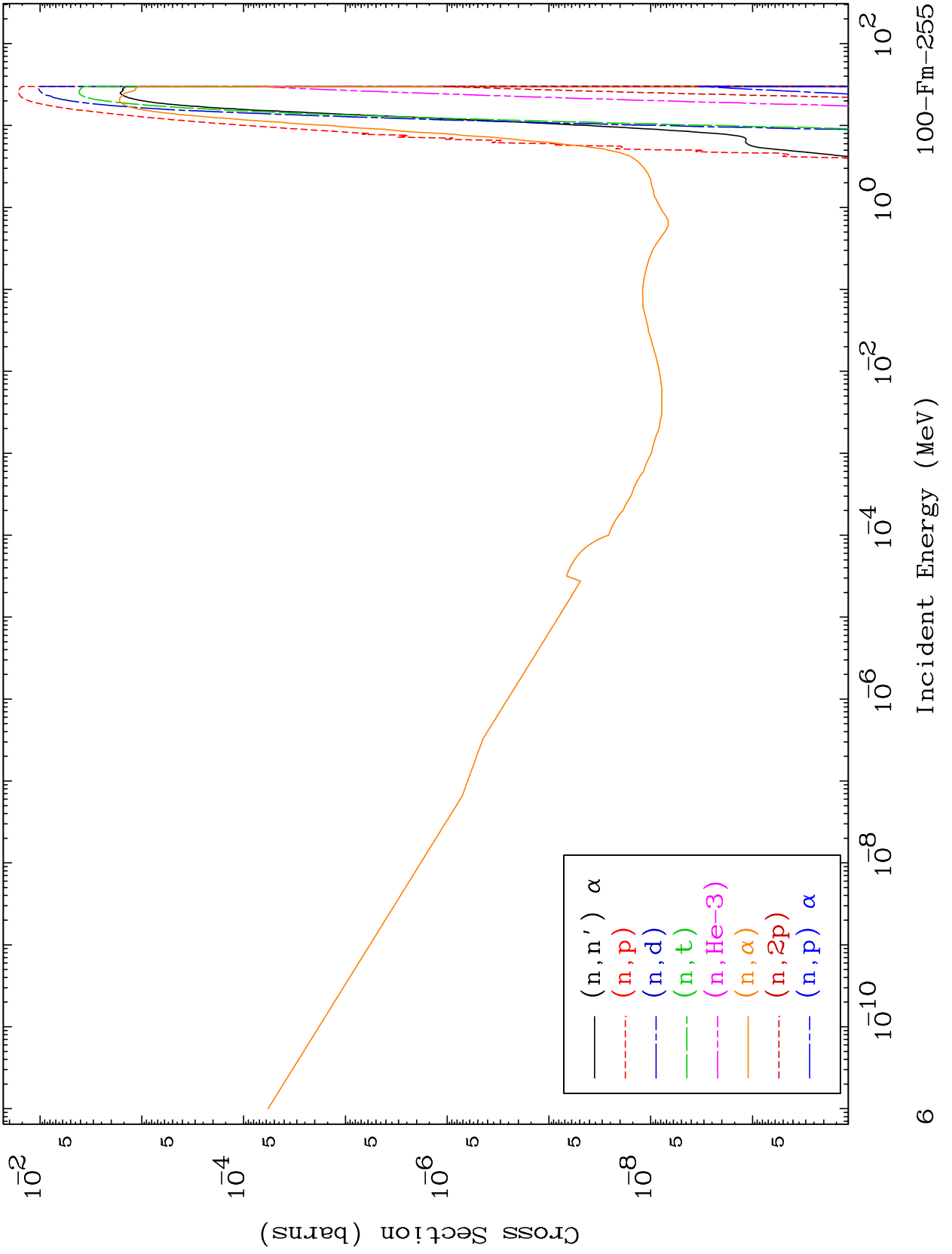
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MAT 9936

Charged Particle
293 Kelvin Cross Sections

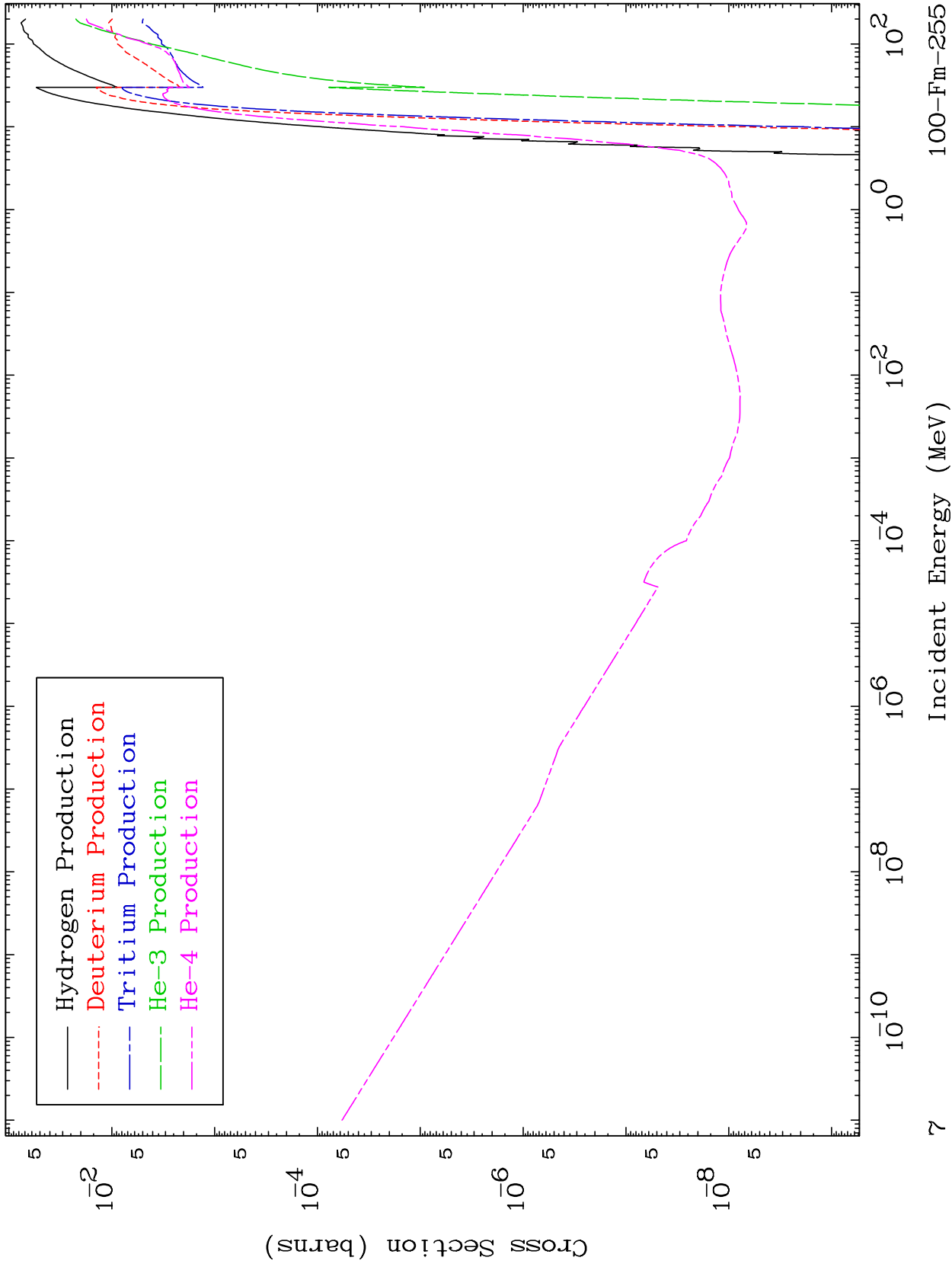
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Particle Production
293 Kelvin Cross Sections

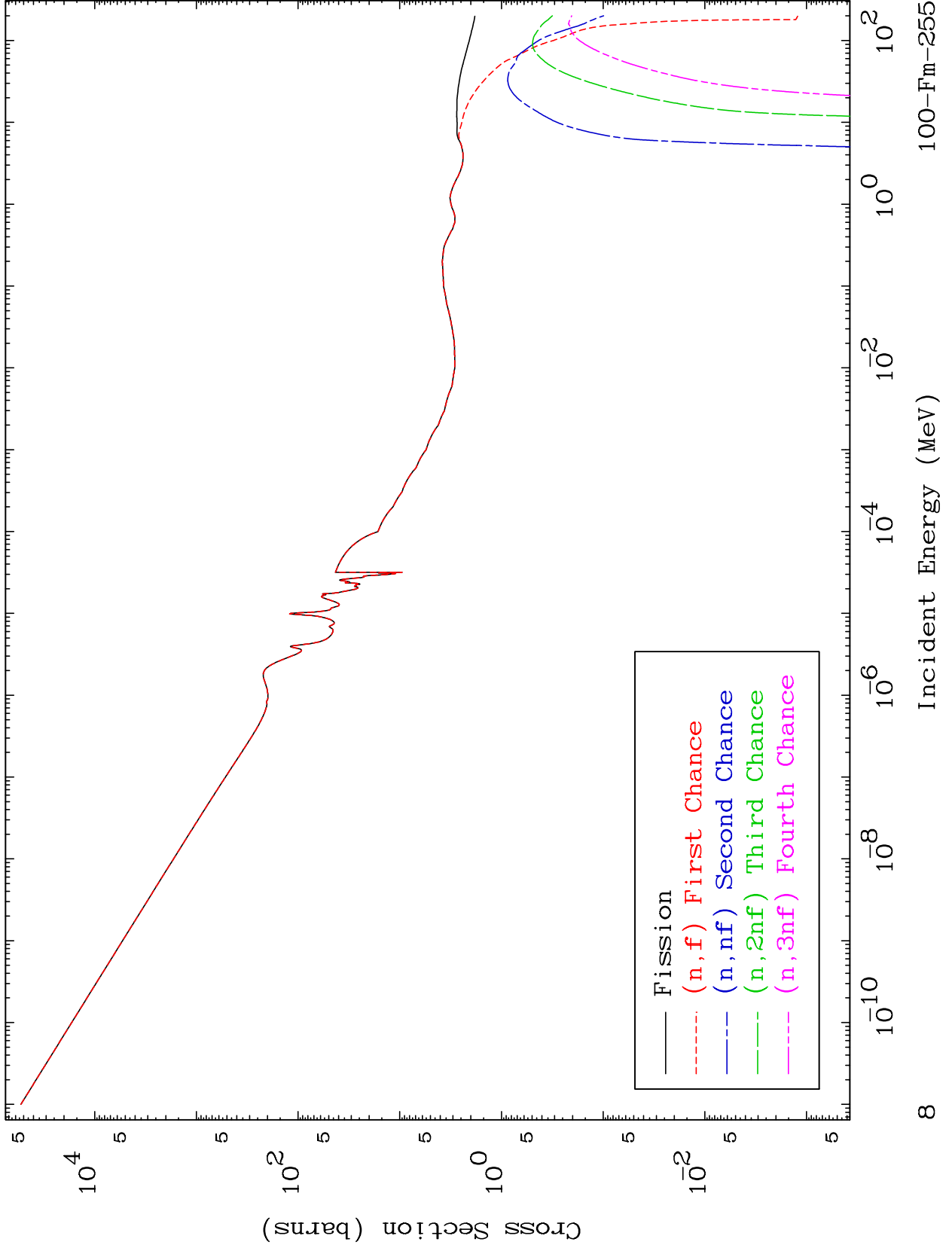
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MAT 9936

Fission
293 Kelvin Cross Sections

100-Fm-255

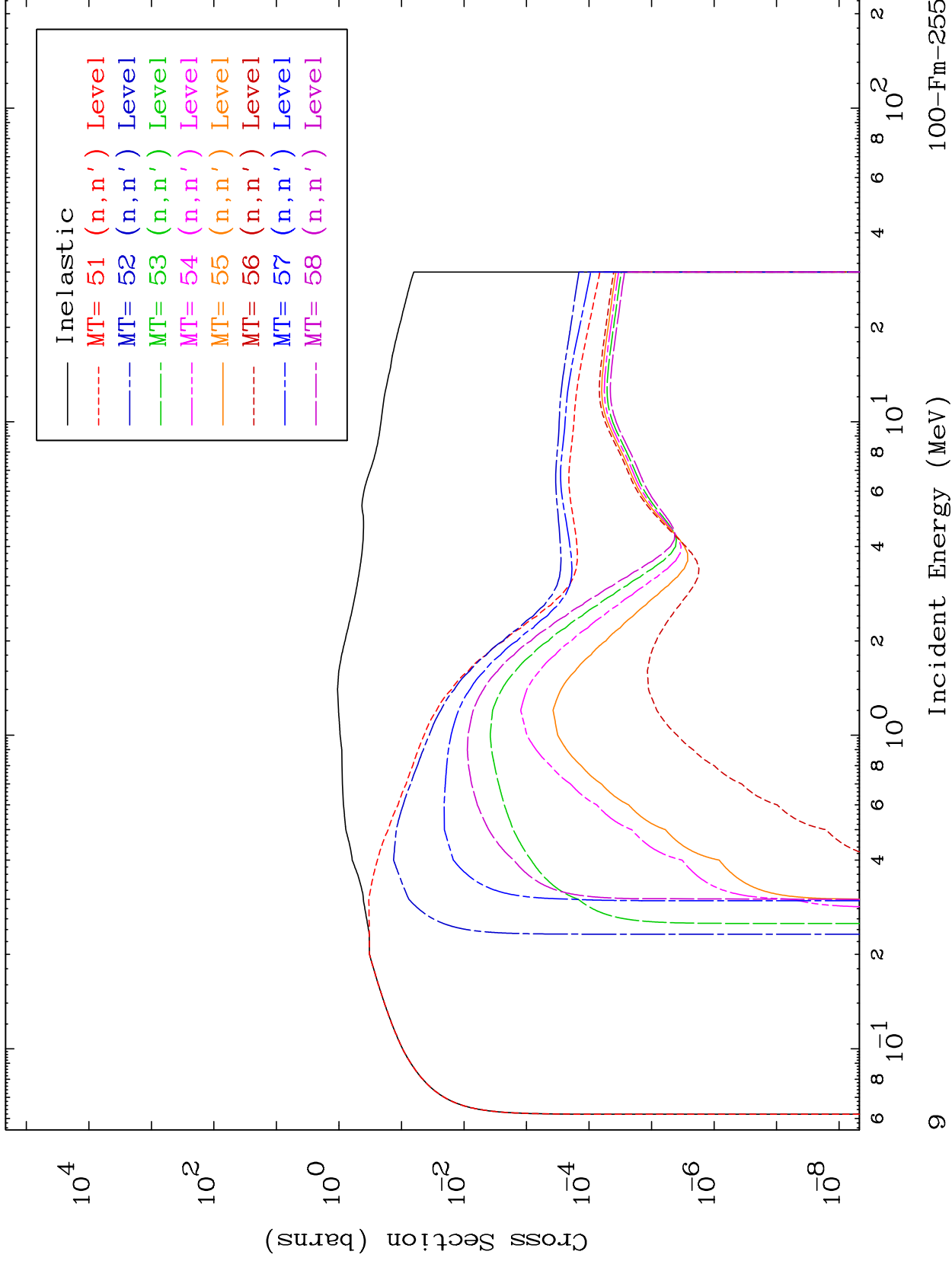


MAT 9936

(n,n') Levels

100-Fm-255

293 Kelvin Cross Sections



9

Incident Energy (MeV)

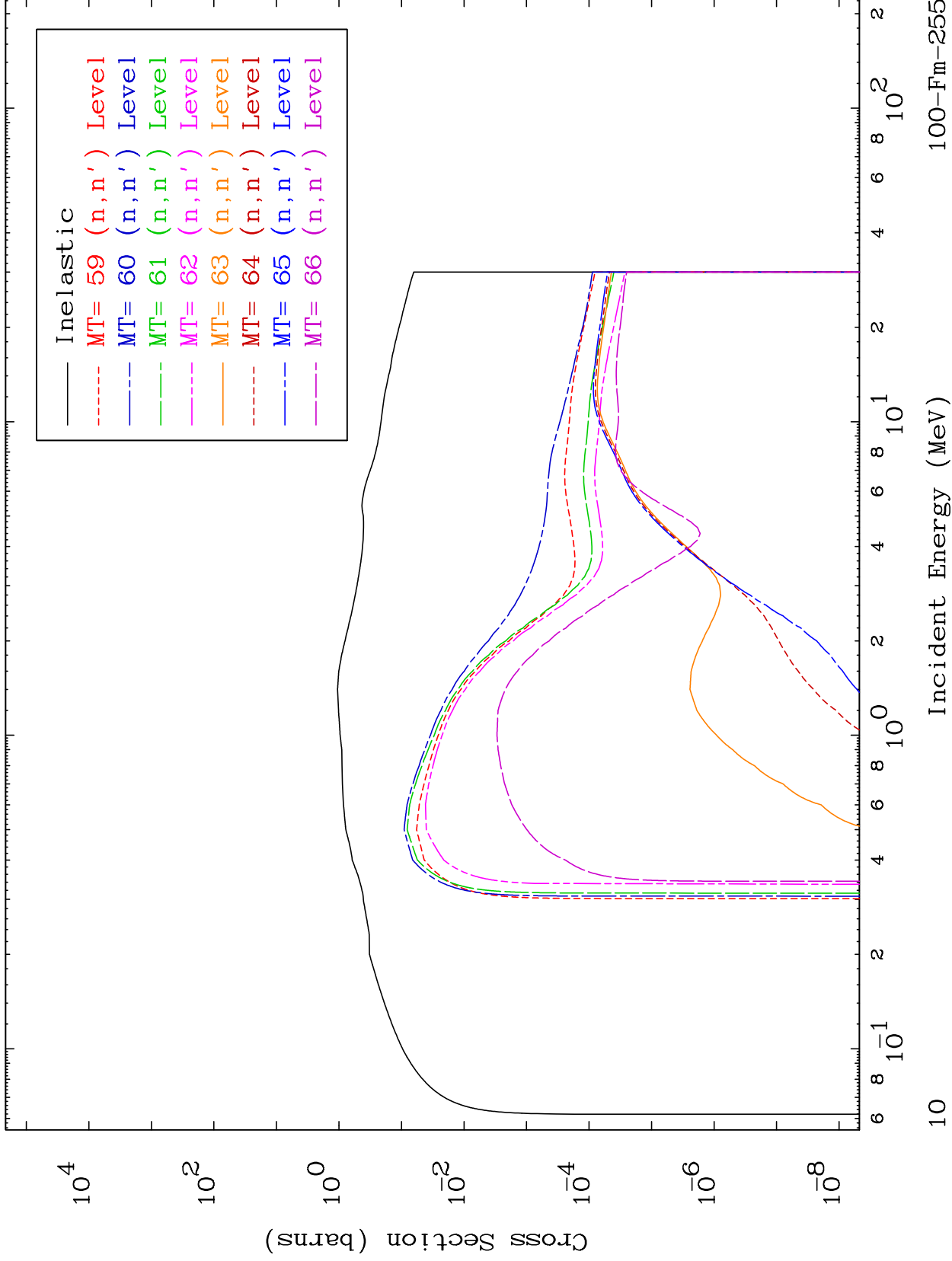
100-Fm-255

MAT 9936

(n,n') Levels

100-Fm-255

293 Kelvin Cross Sections



10

Incident Energy (MeV)

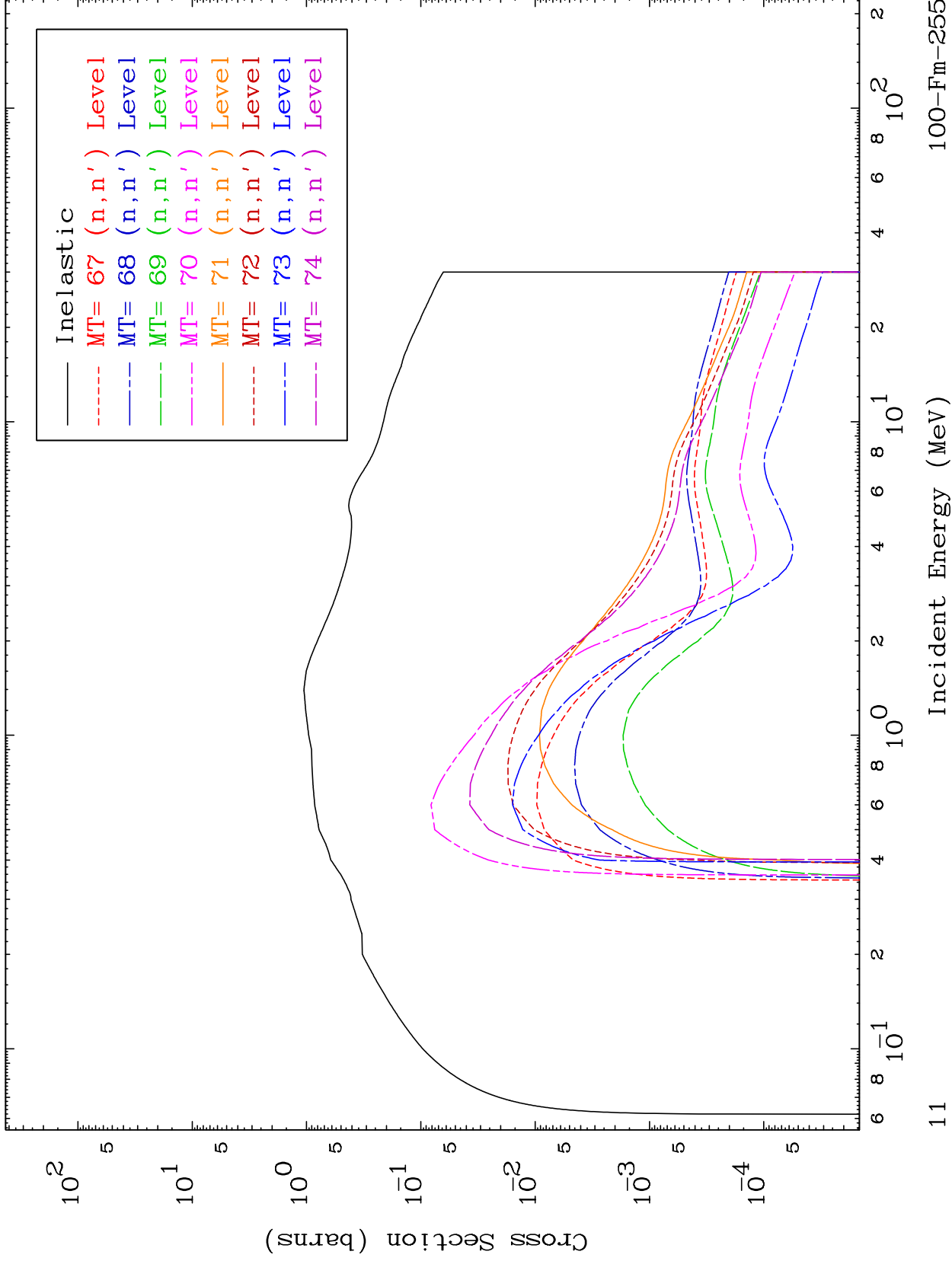
100-Fm-255

MAT 9936

(n,n') Levels

100-Fm-255

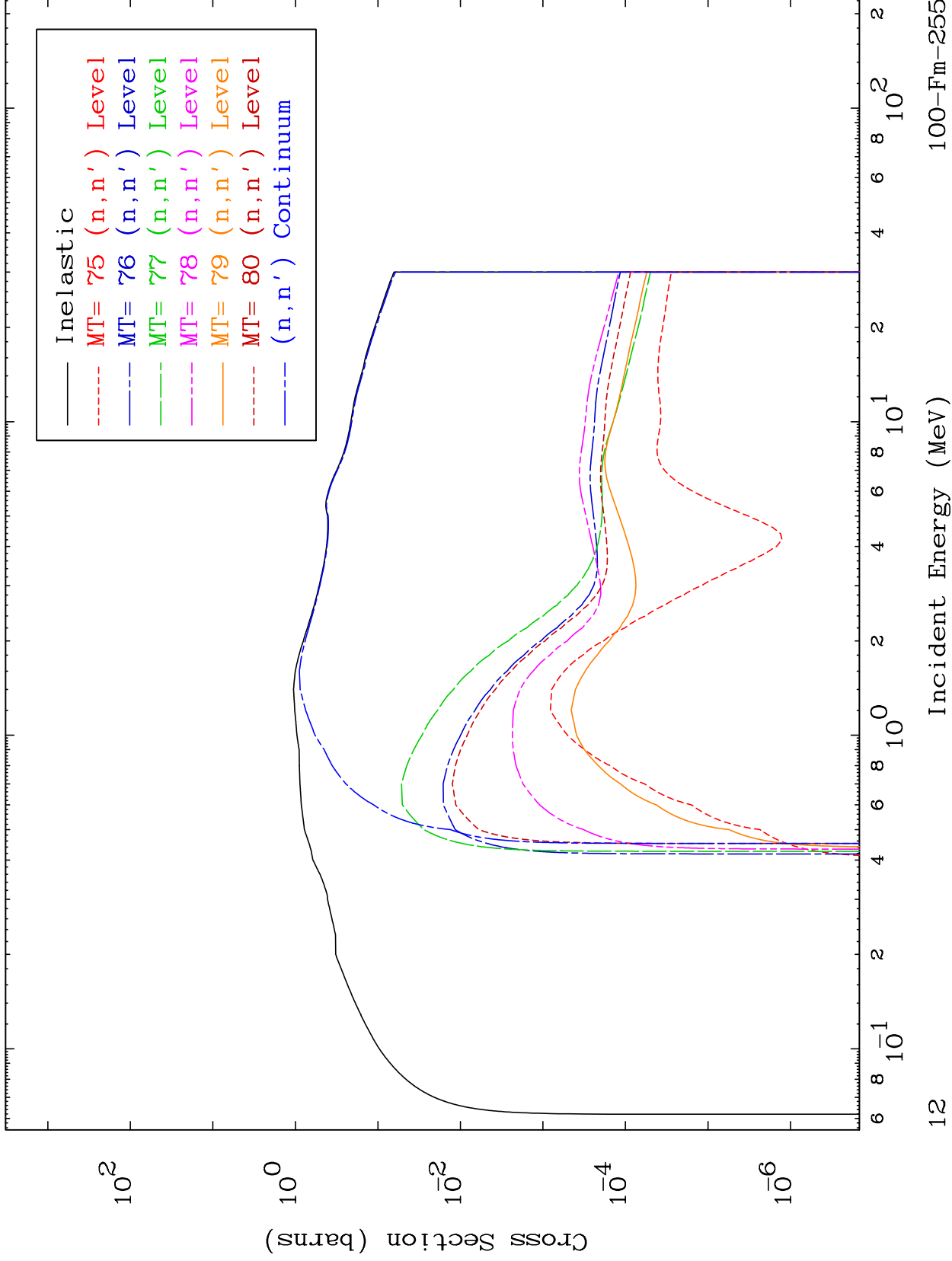
293 Kelvin Cross Sections



MAT 9936

(n,n') Levels
293 Kelvin Cross Sections

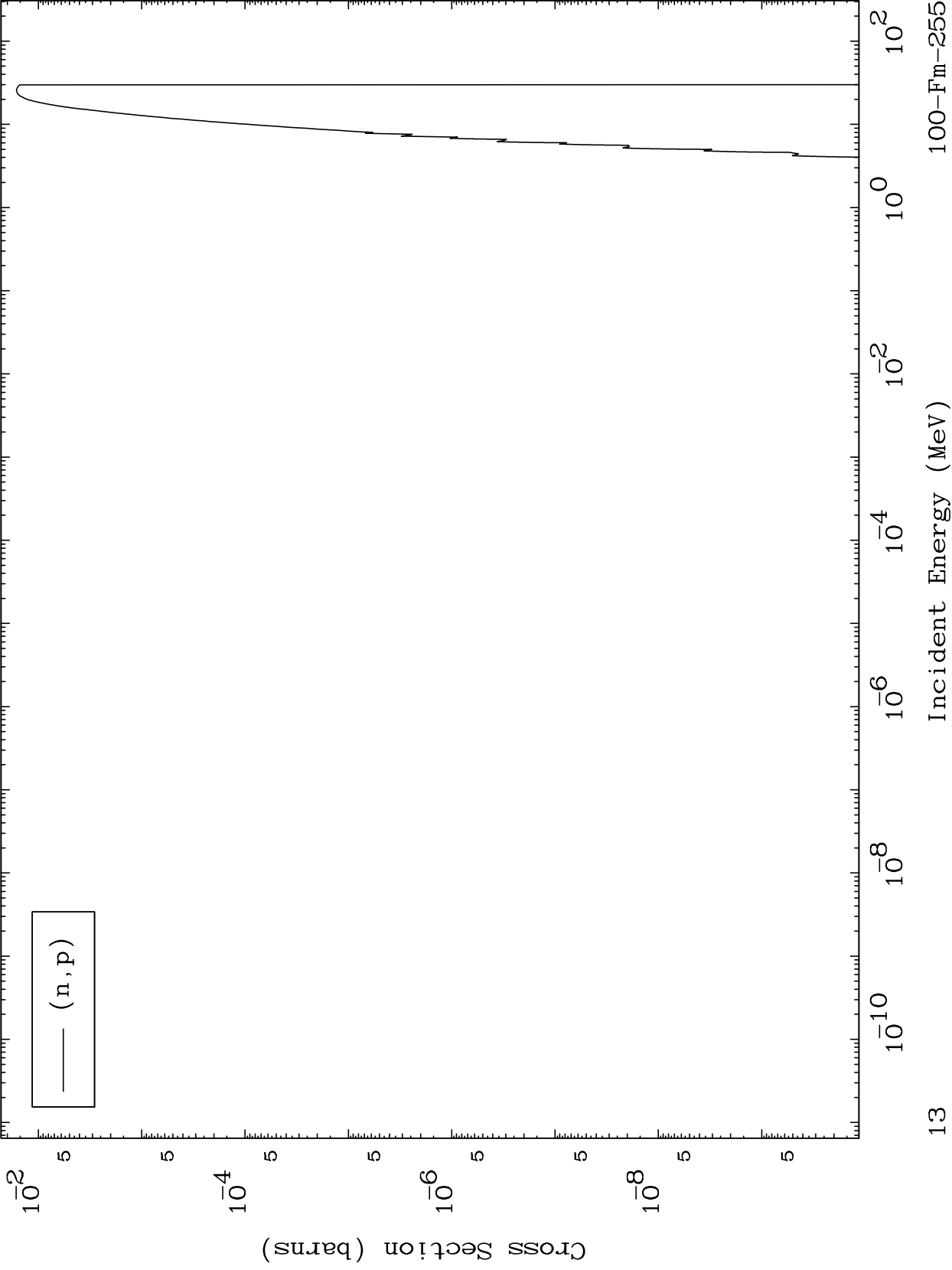
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(n,p) Levels
293 Kelvin Cross Sections

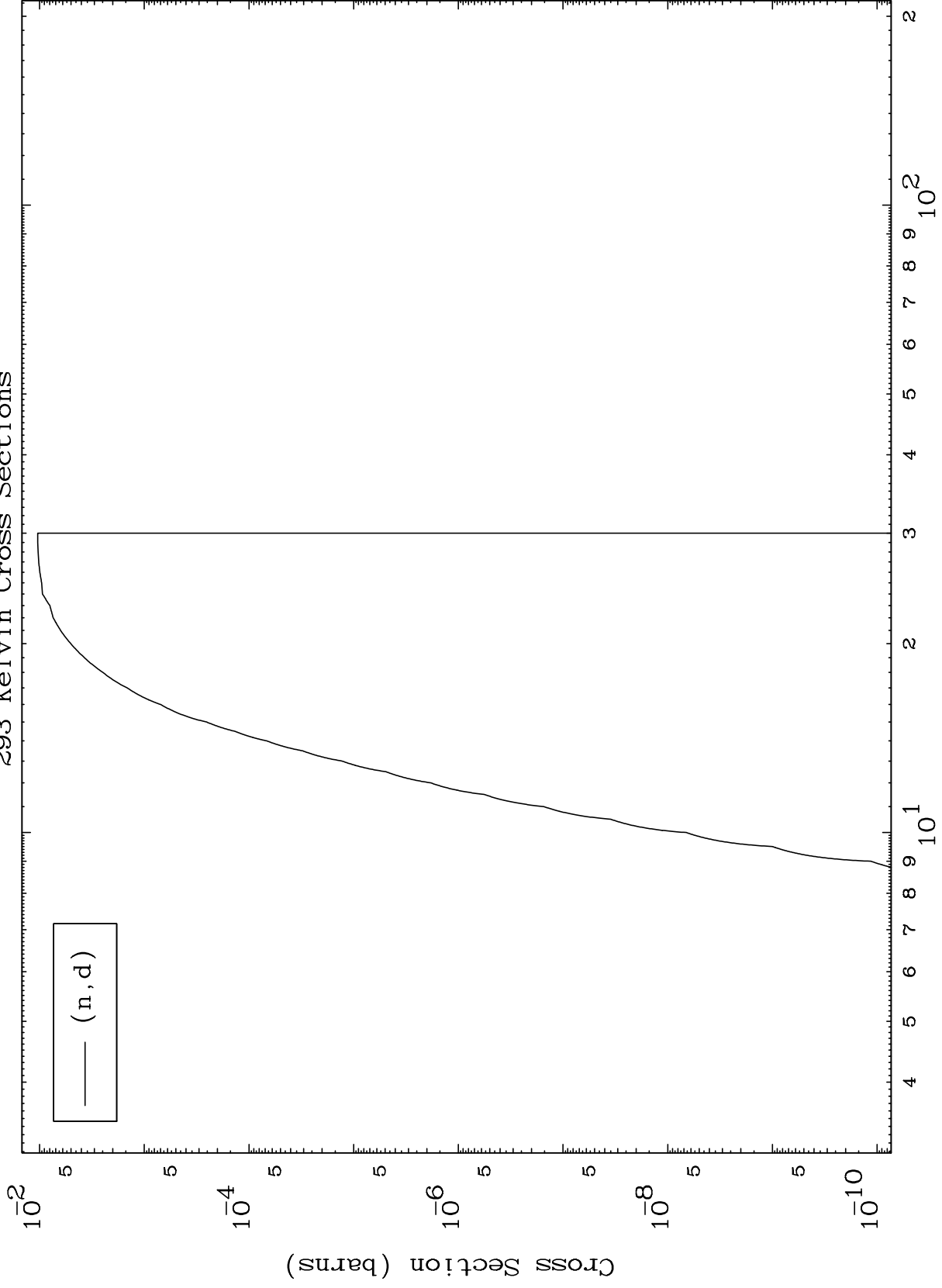
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MAT 9936

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-255



14

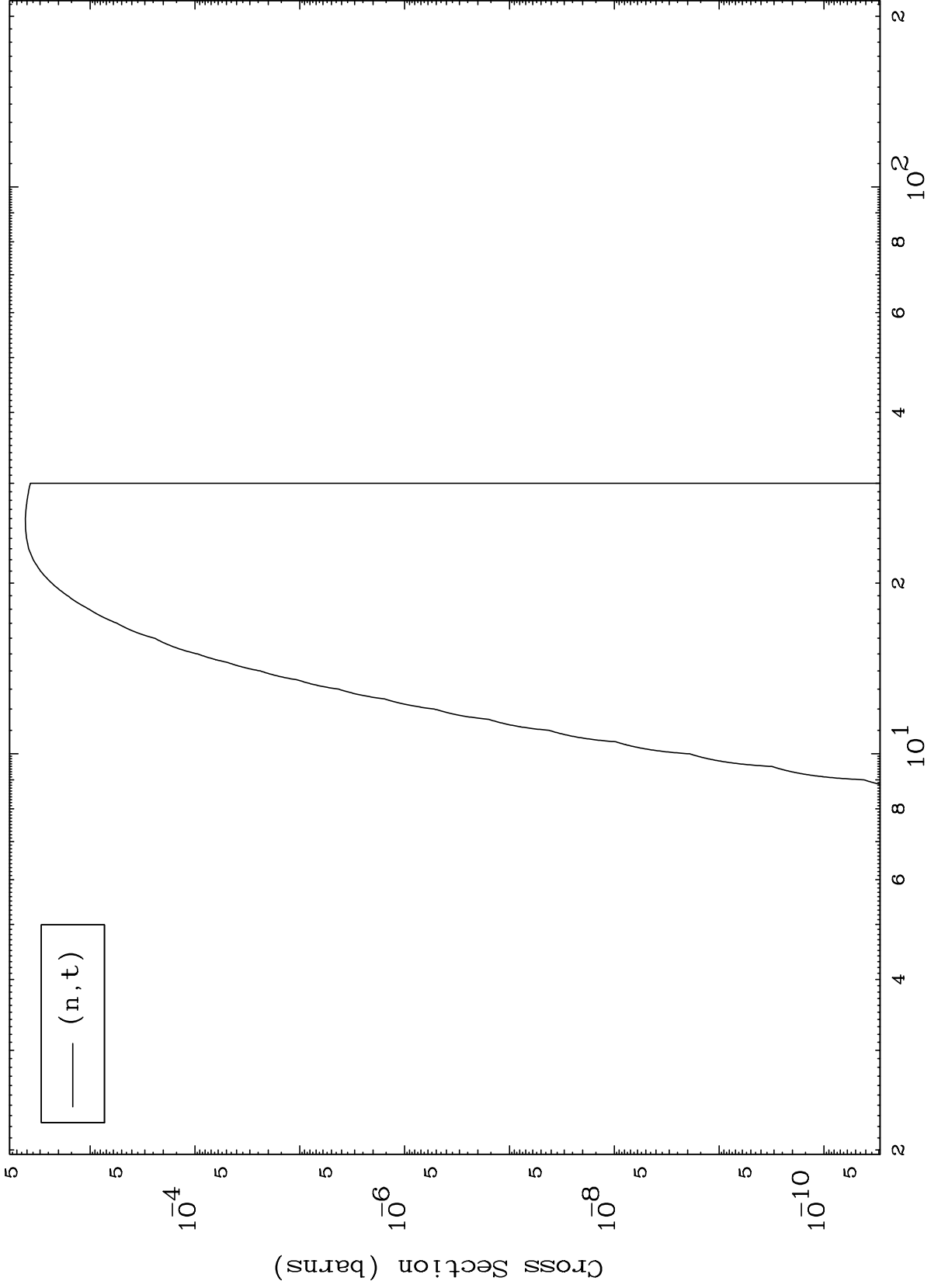
Incident Energy (MeV)

100-Fm-255

MAT 9936

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-255



(n,t)

15

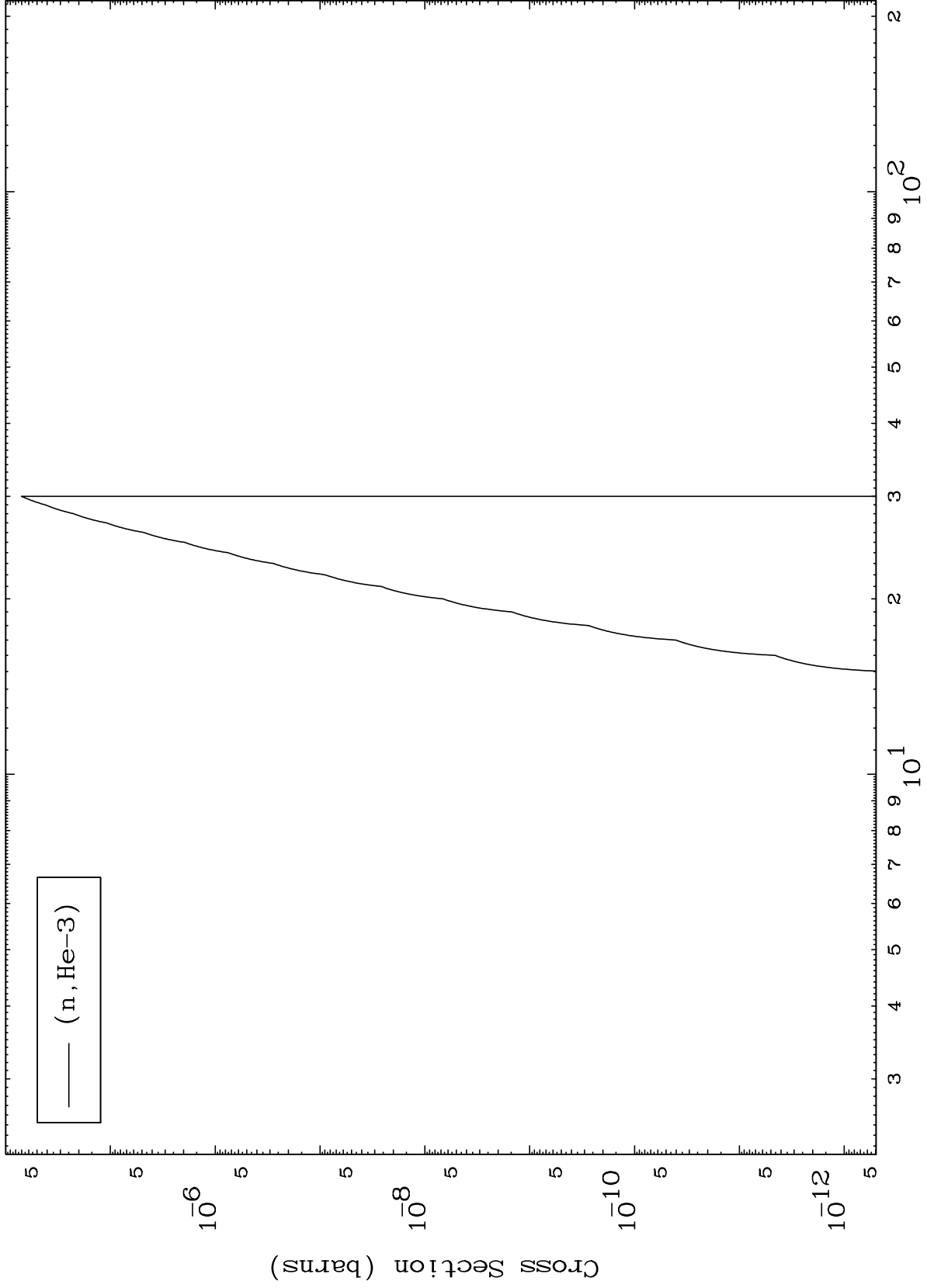
Incident Energy (MeV)

100-Fm-255

MAT 9936

(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-255



16

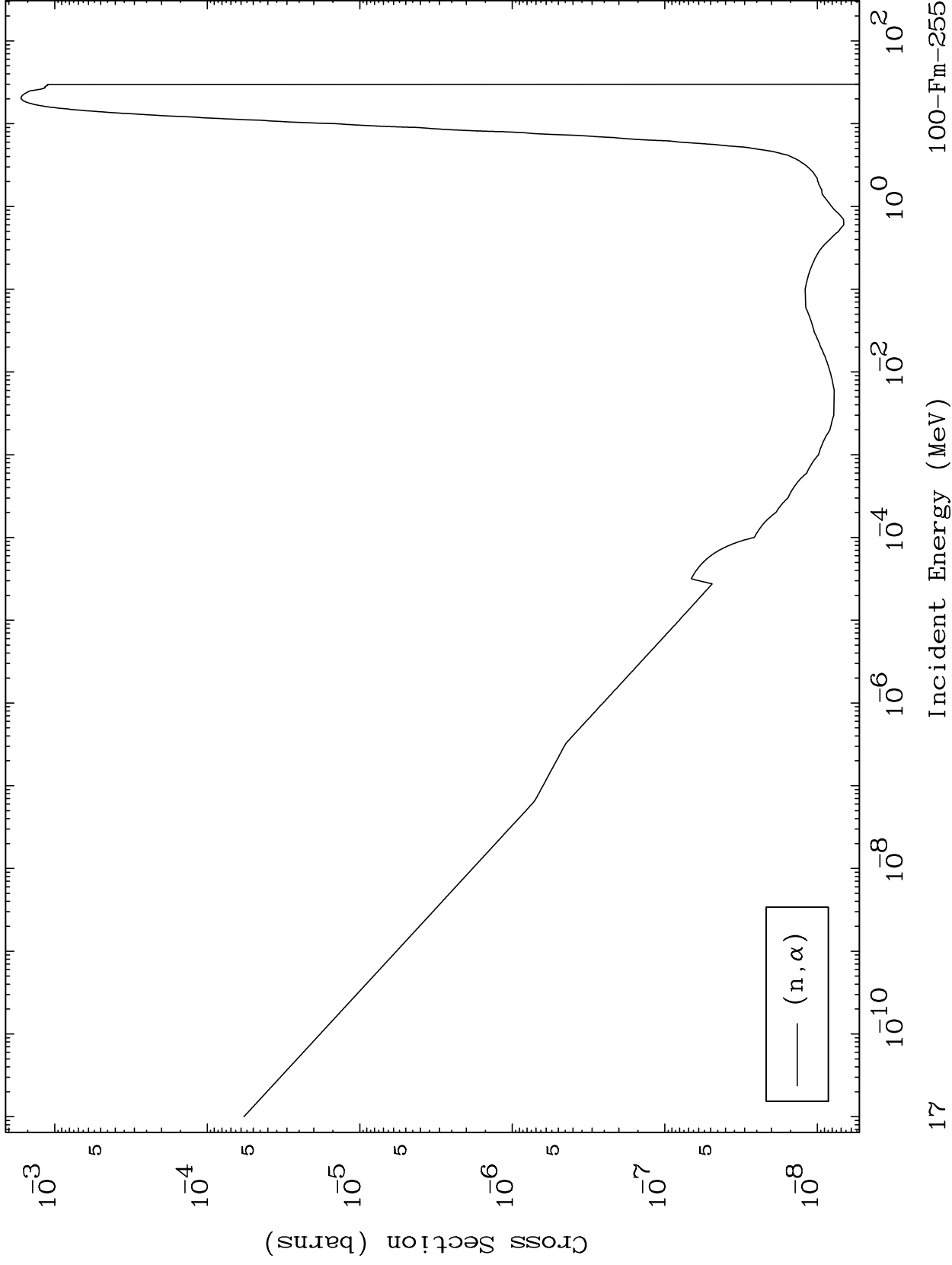
Incident Energy (MeV)

100-Fm-255

MAT 9936

(n, α) Levels
293 Kelvin Cross Sections

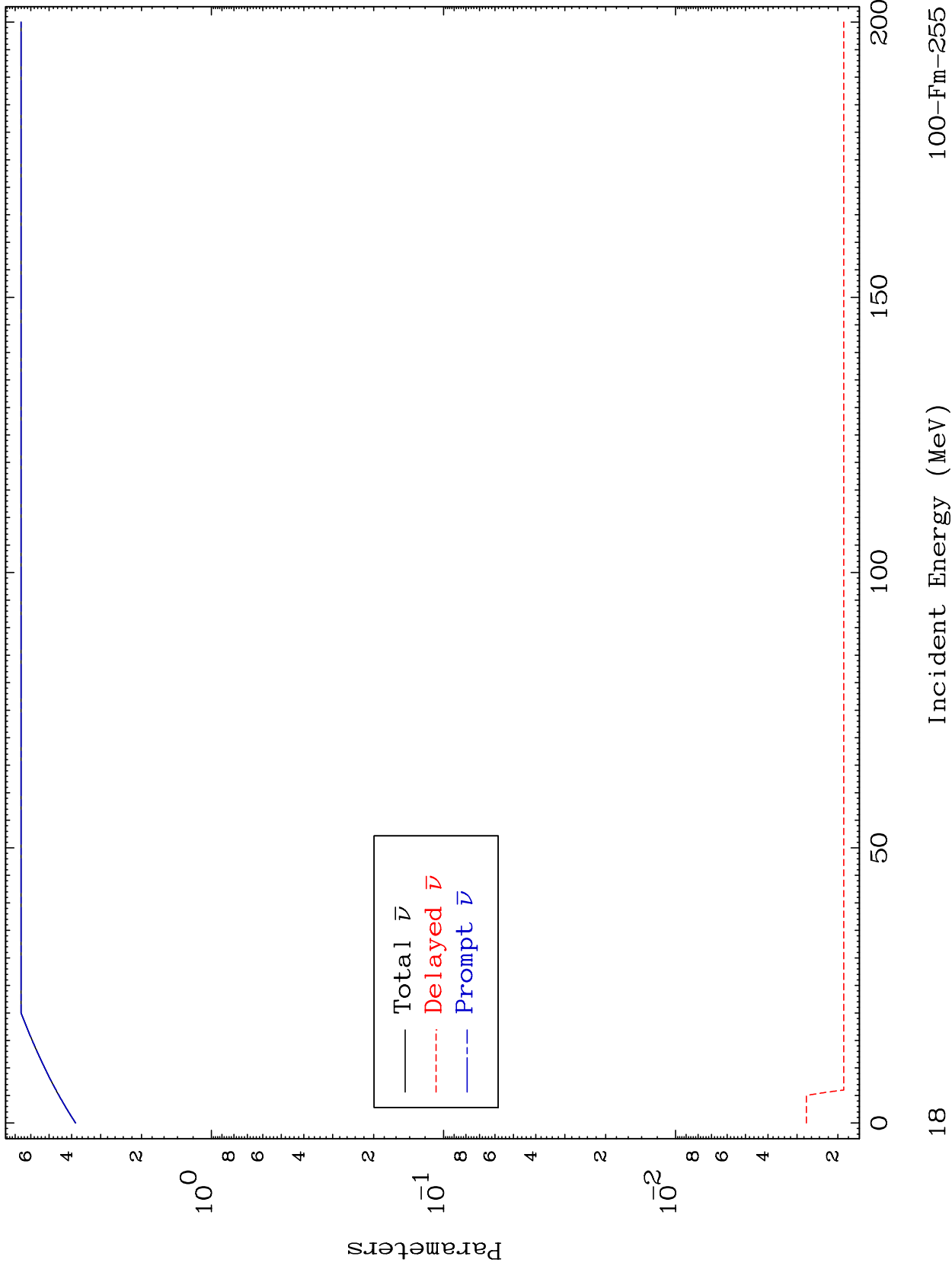
100-Fm-255



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Energy Release
Parameters

100-Fm-255



18

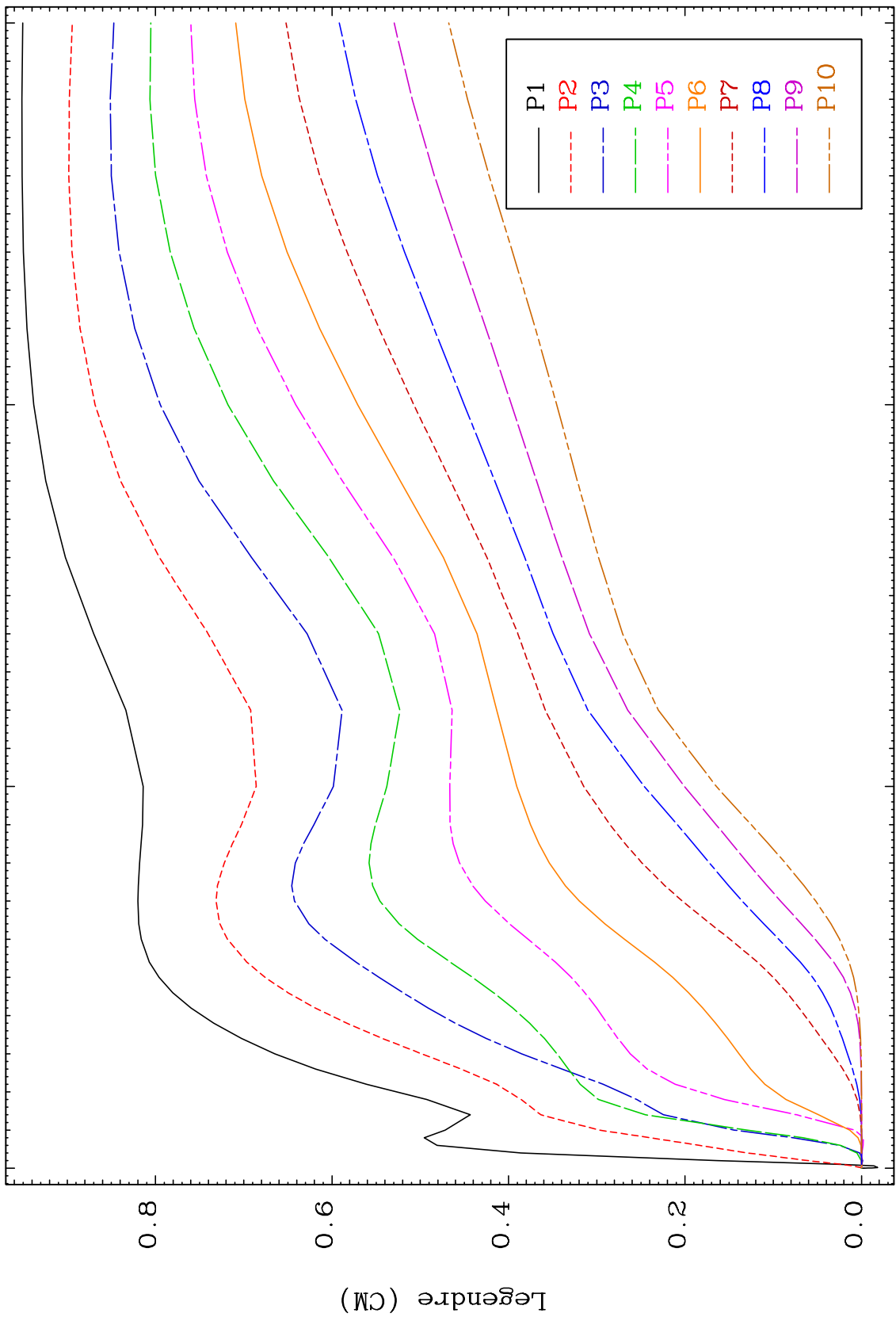
Incident Energy (MeV)

100-Fm-255

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Elastic Legendre Coefficients

100-Fm-255



19

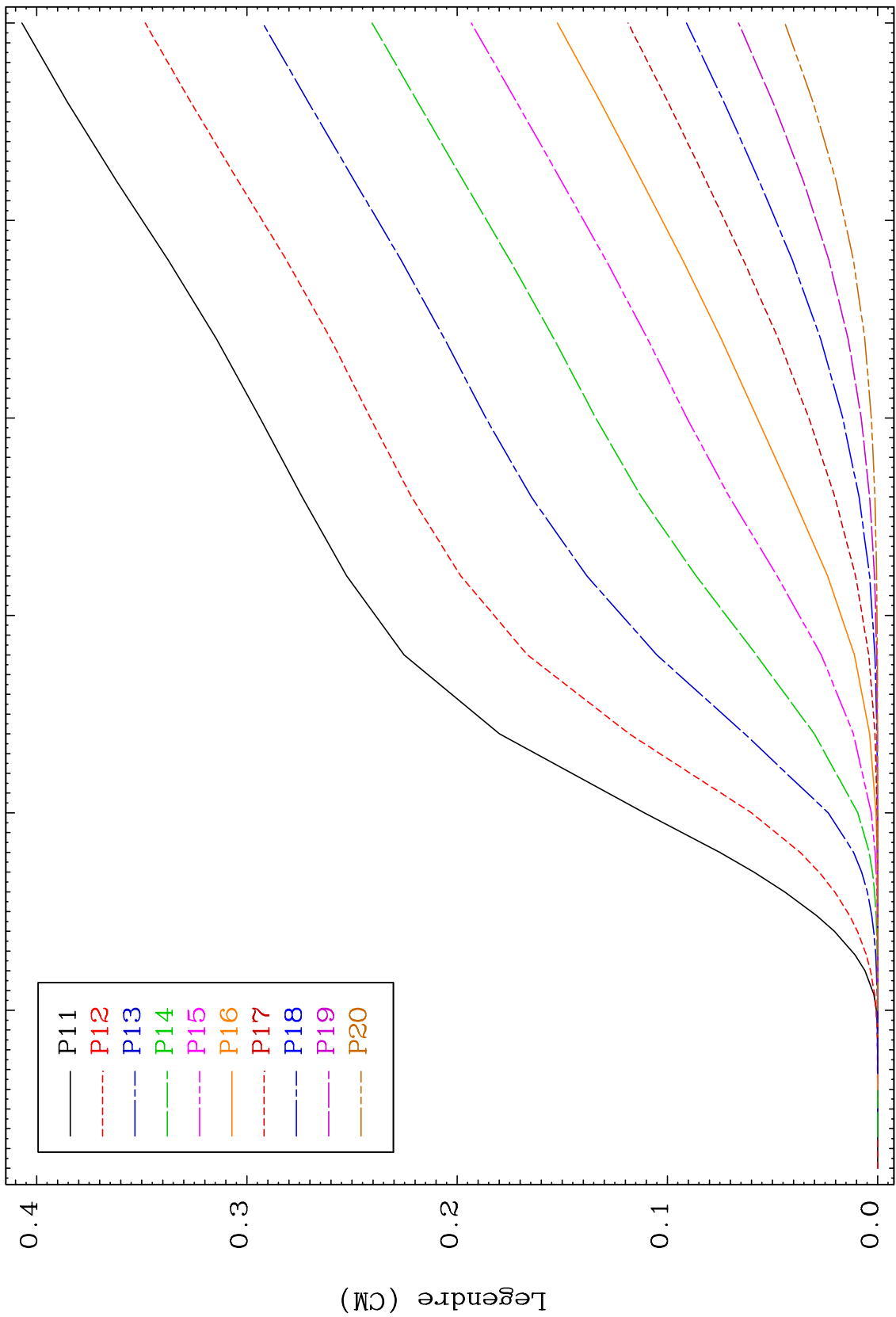
Incident Energy (MeV)

100-Fm-255

MAT 9936

Elastic Legendre Coefficients

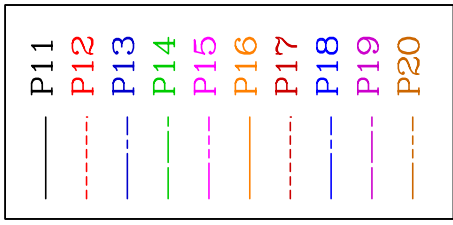
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20

Incident Energy (MeV)

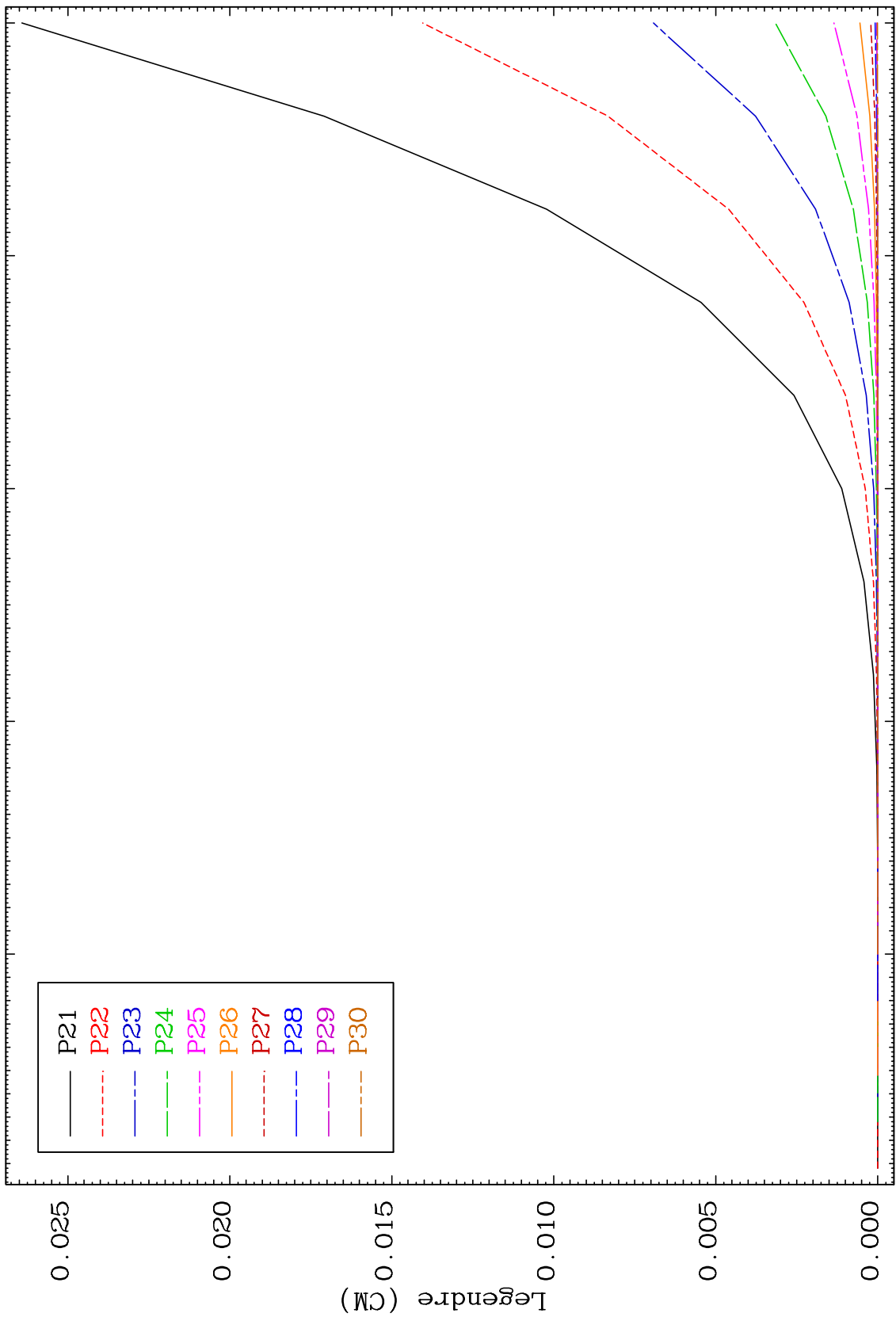
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Elastic Legendre Coefficients

100-Fm-255



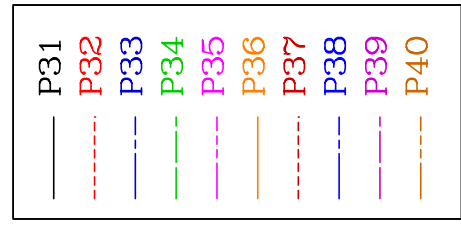
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Incident Energy (MeV)

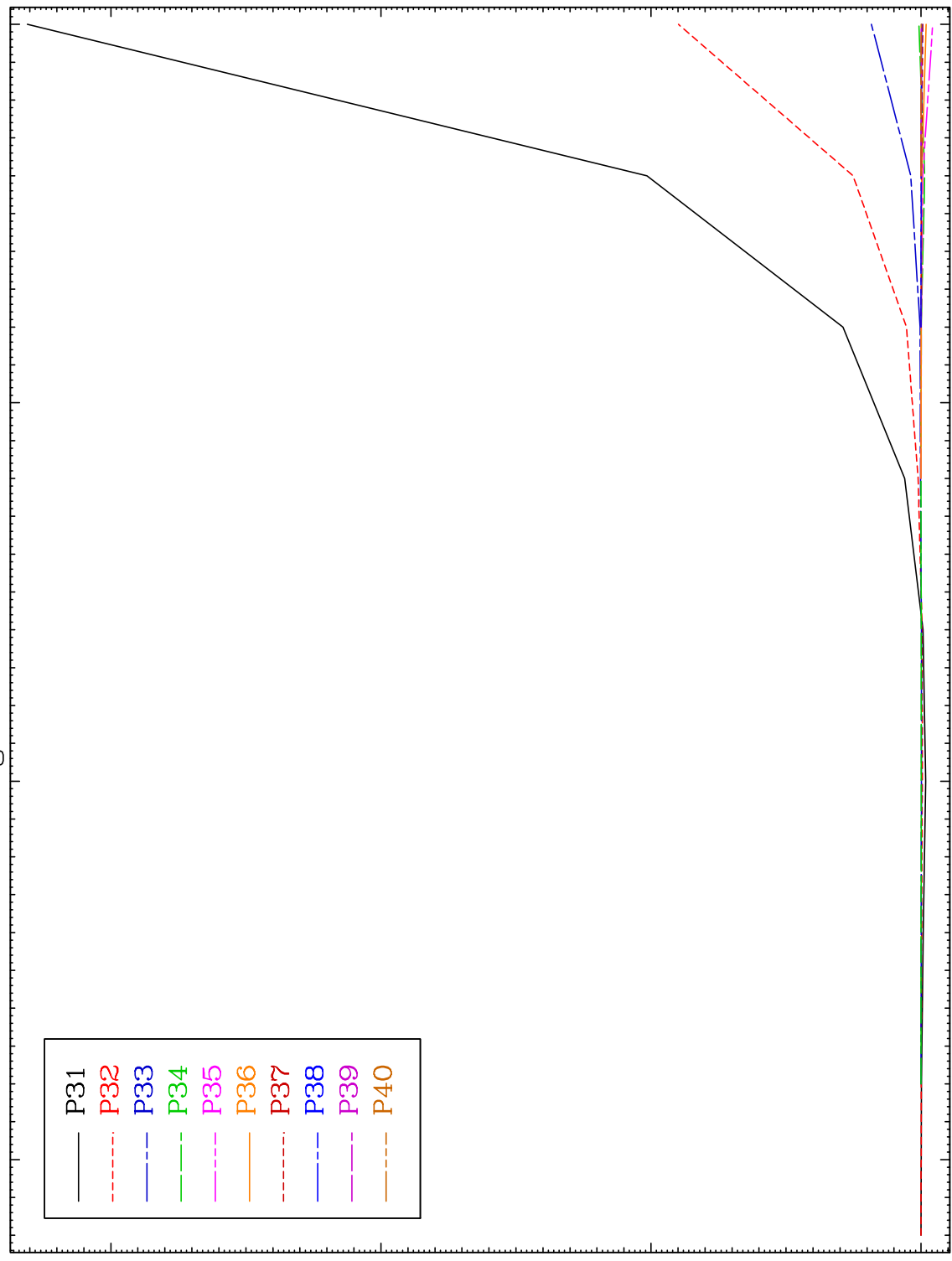
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Elastic Legendre Coefficients

100-Fm-255



Legendre (CM)
 $\times 10^{-6}$

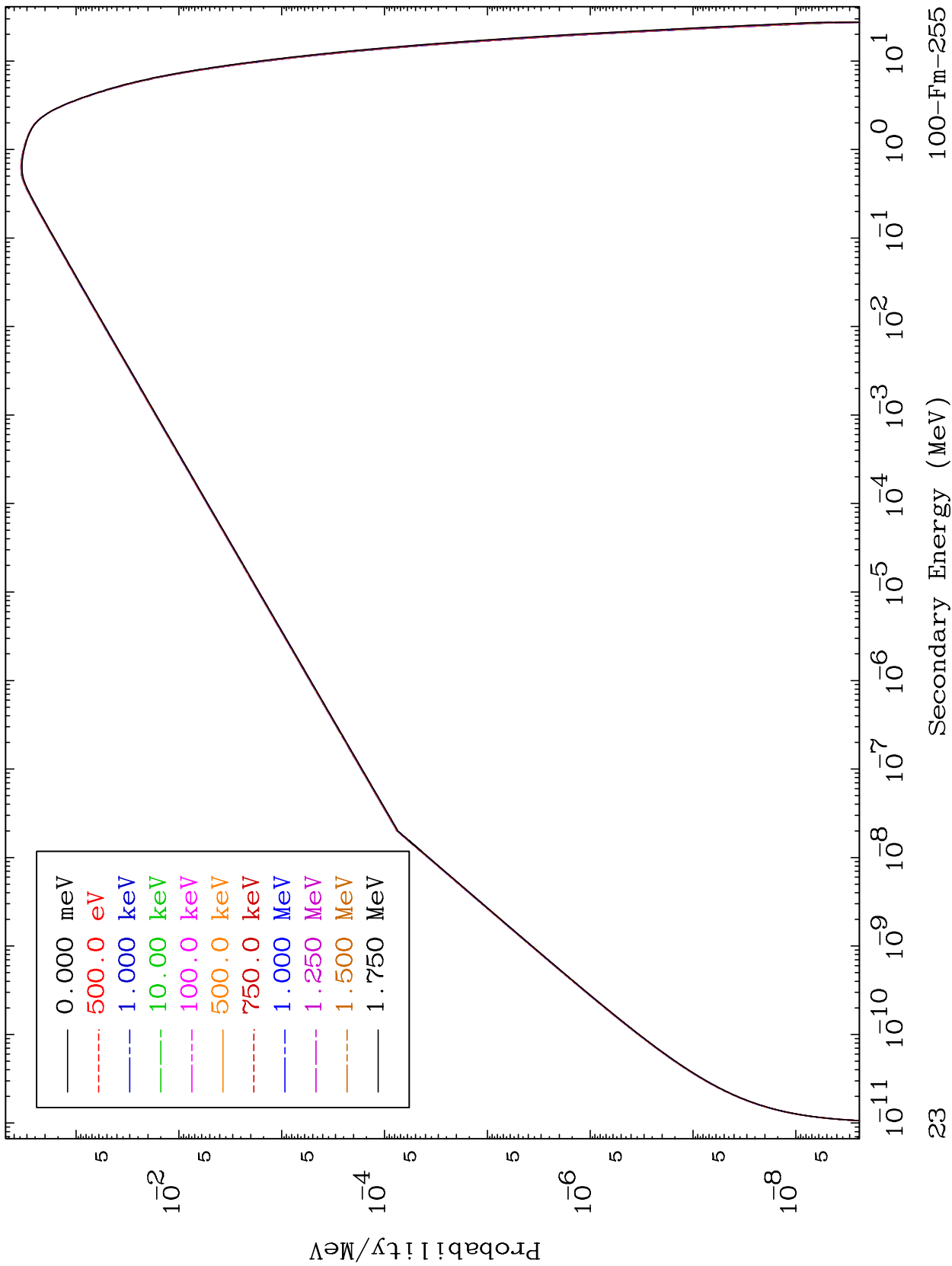


22

Incident Energy (MeV)

100-Fm-255

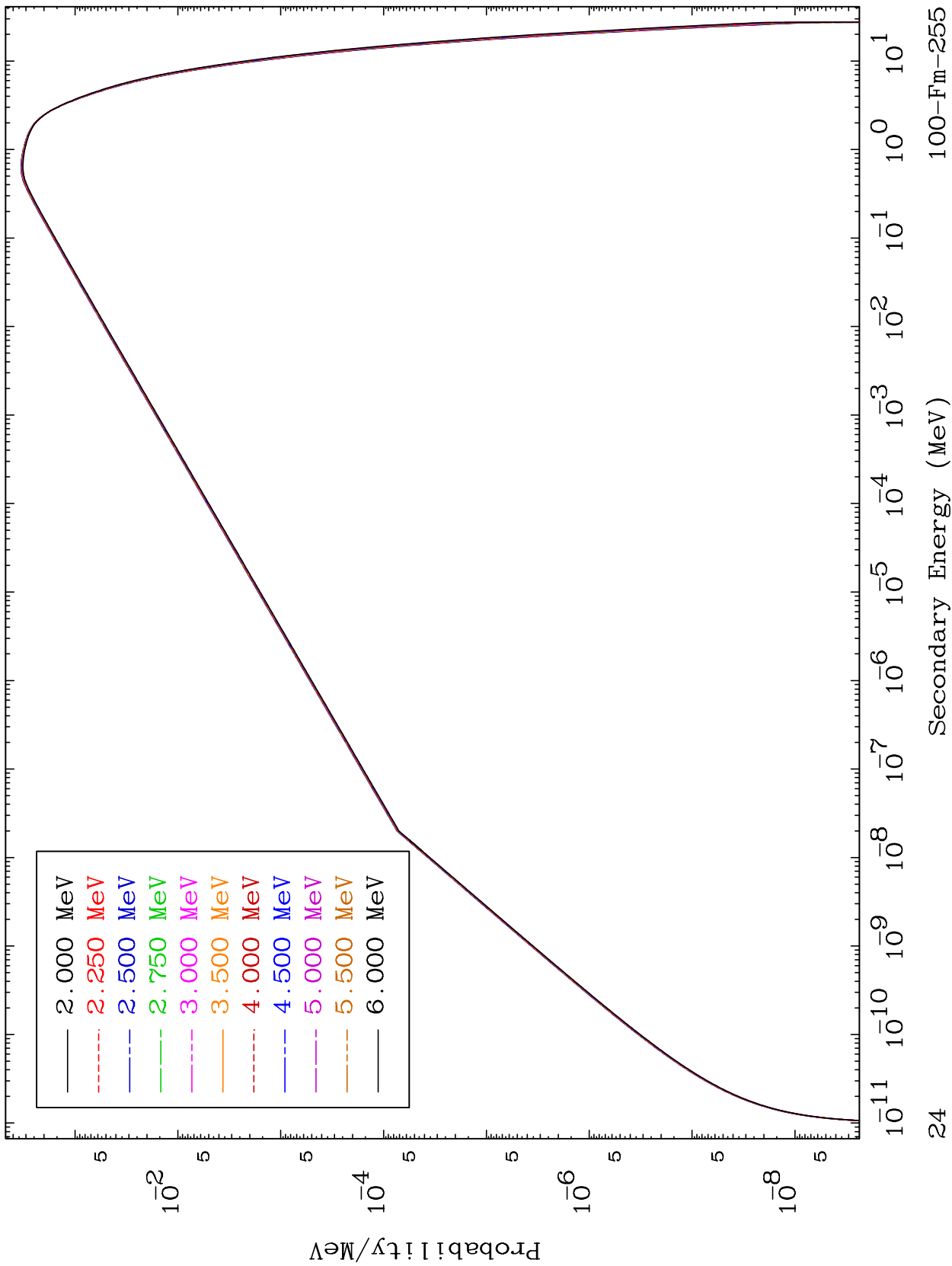
Fission Energy Distributions



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Fission Energy Distributions

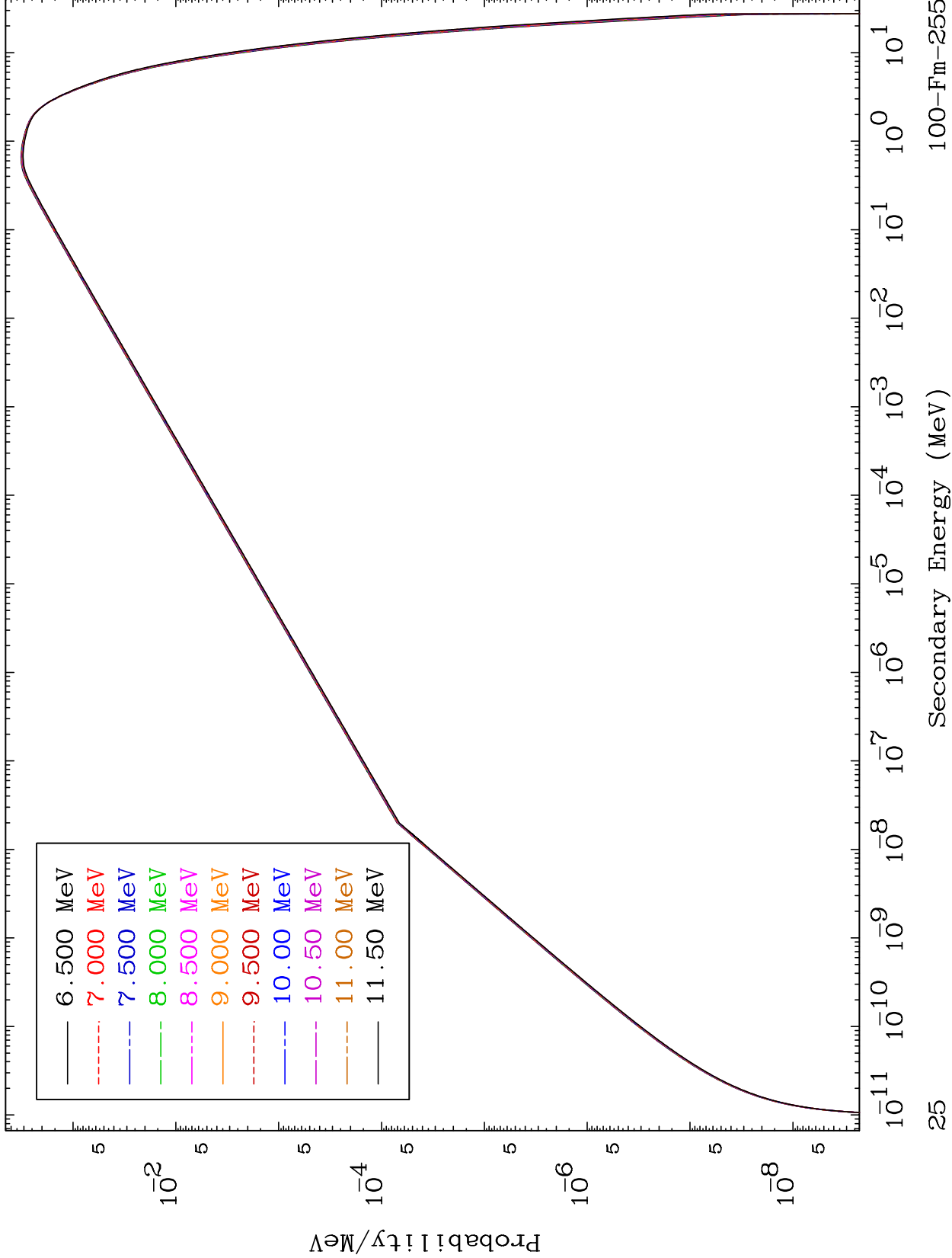
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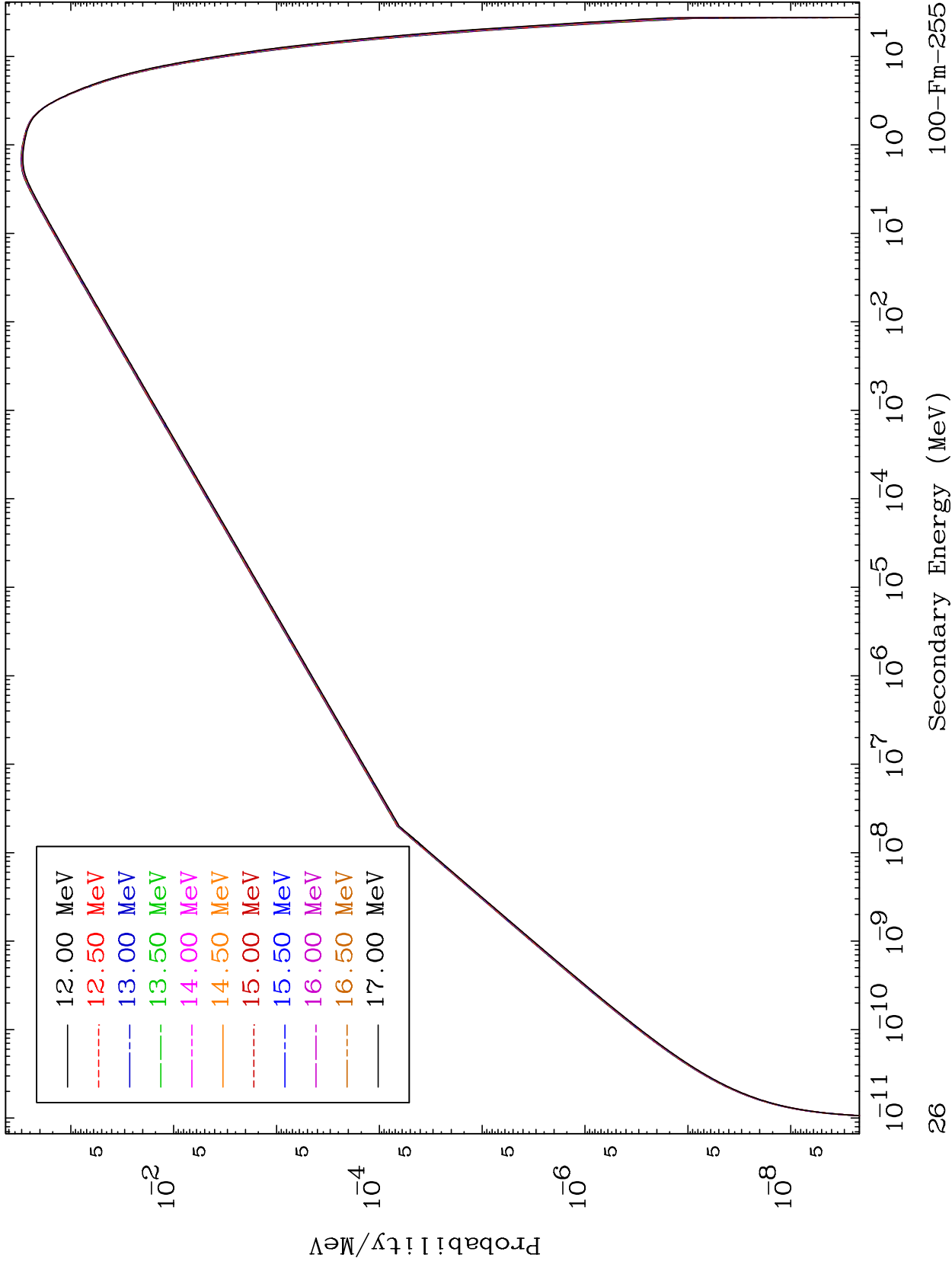
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Fission Energy Distributions

100-Fm-255



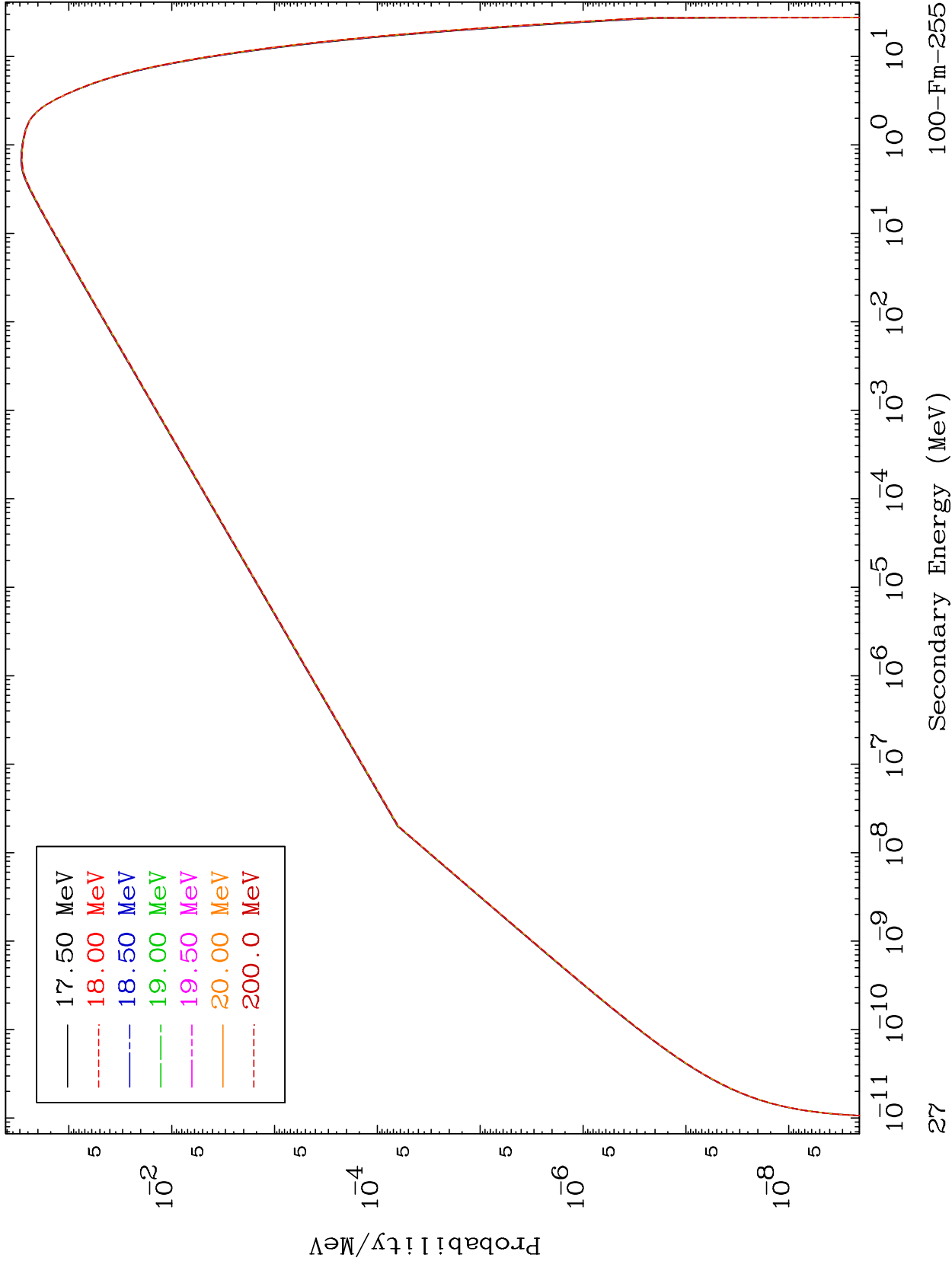
Fission Energy Distributions



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Fission Energy Distributions

100-Fm-255

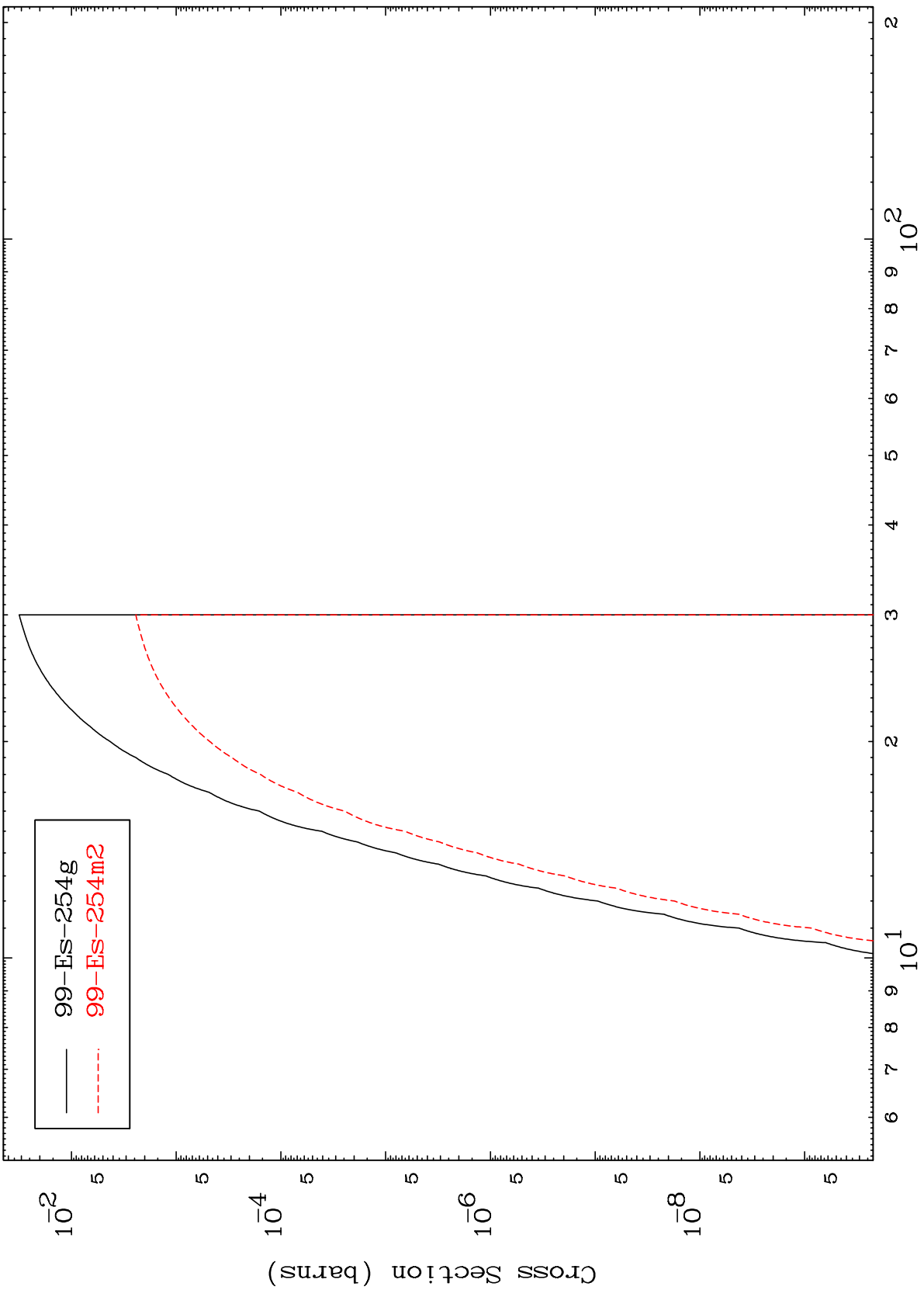


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(n,n') p

100-Fm-255

Radionuclide Production Cross Section



— 99-Es-254g
- - - 99-Es-254m2

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(n,d)

100-Fm-255

