

Program EVALPLOT
(Version 2021-1)

by

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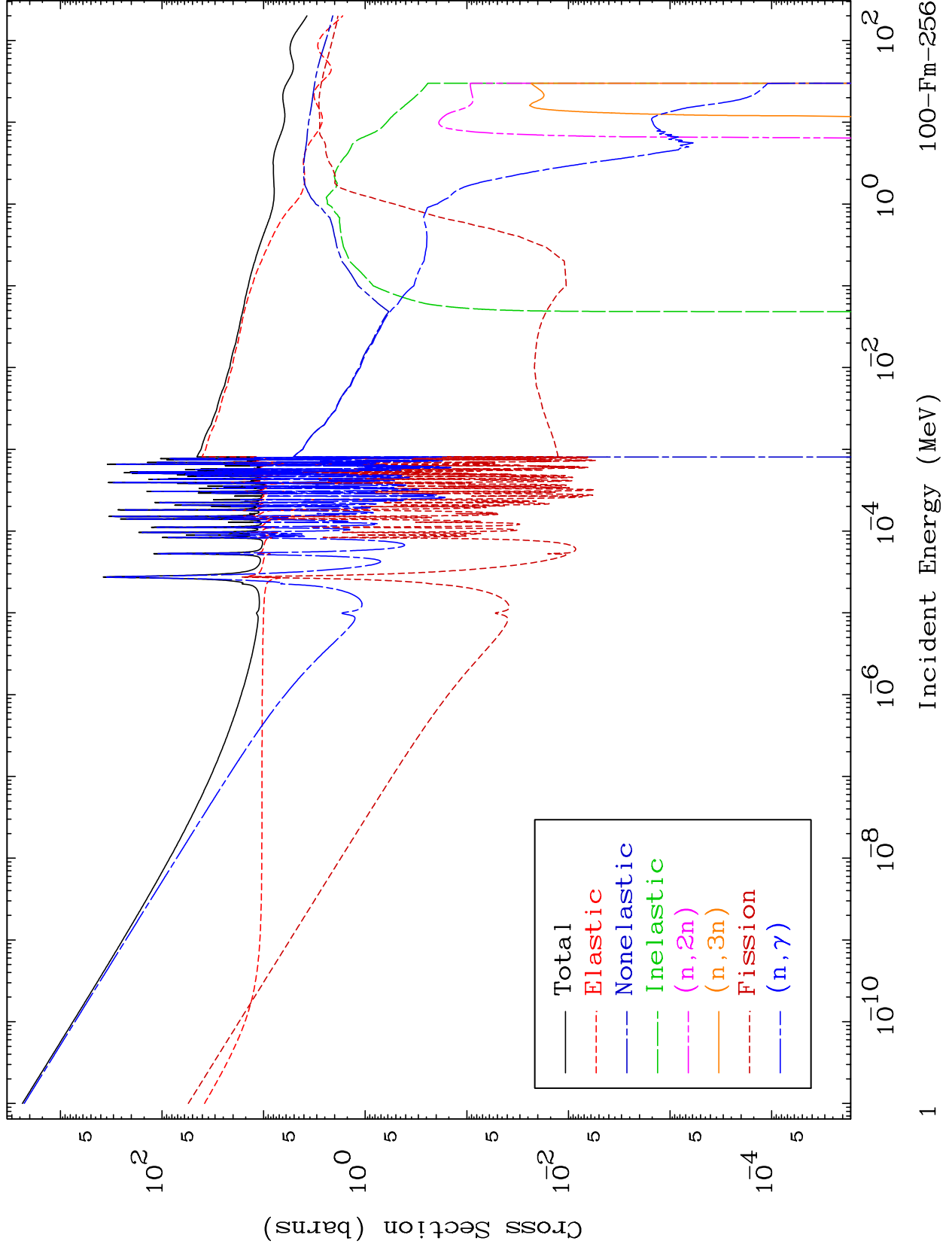
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9937

Neutron Major
293 Kelvin Cross Sections

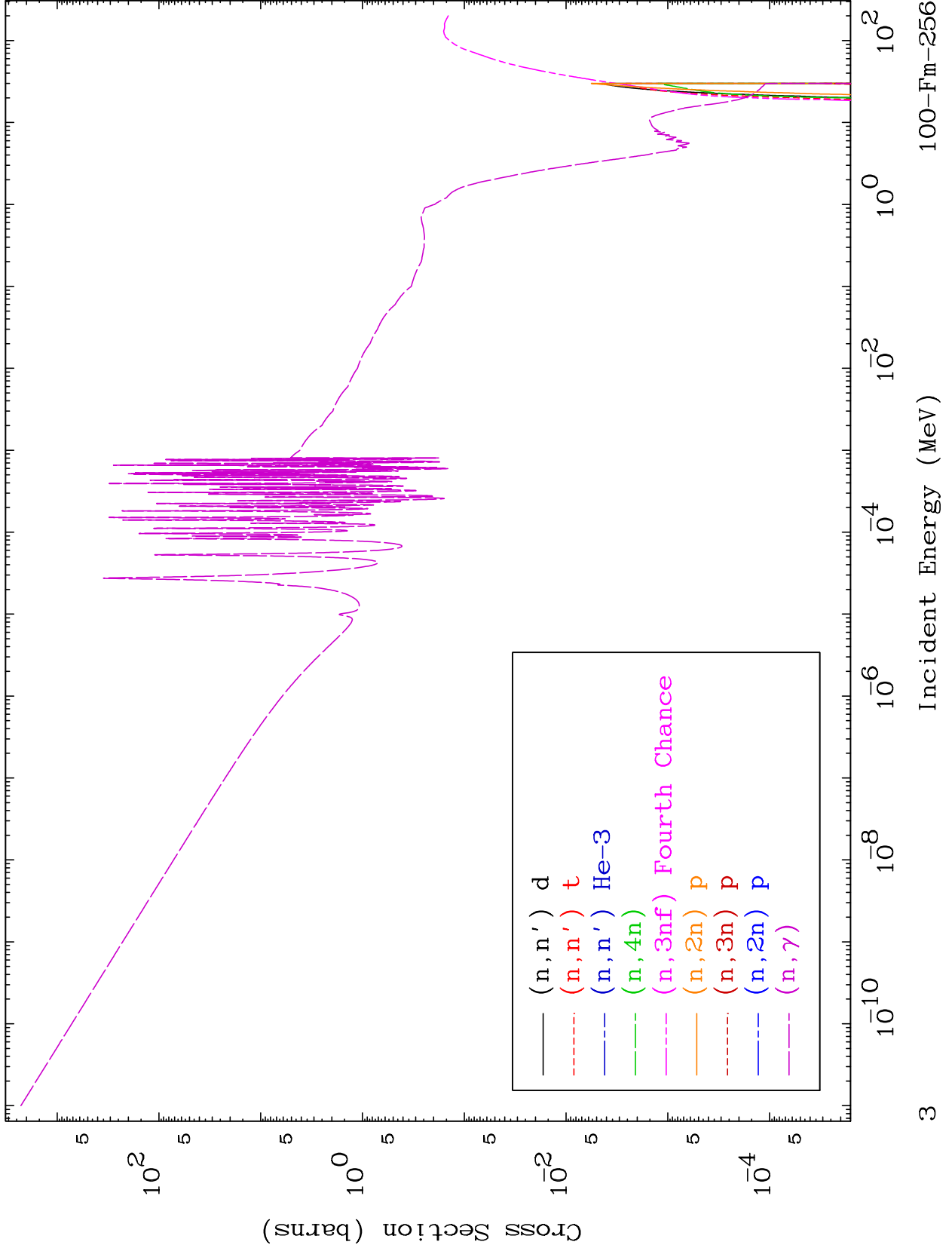
100-Fm-256



MAT 9937

Neutron Absorption
293 Kelvin Cross Sections

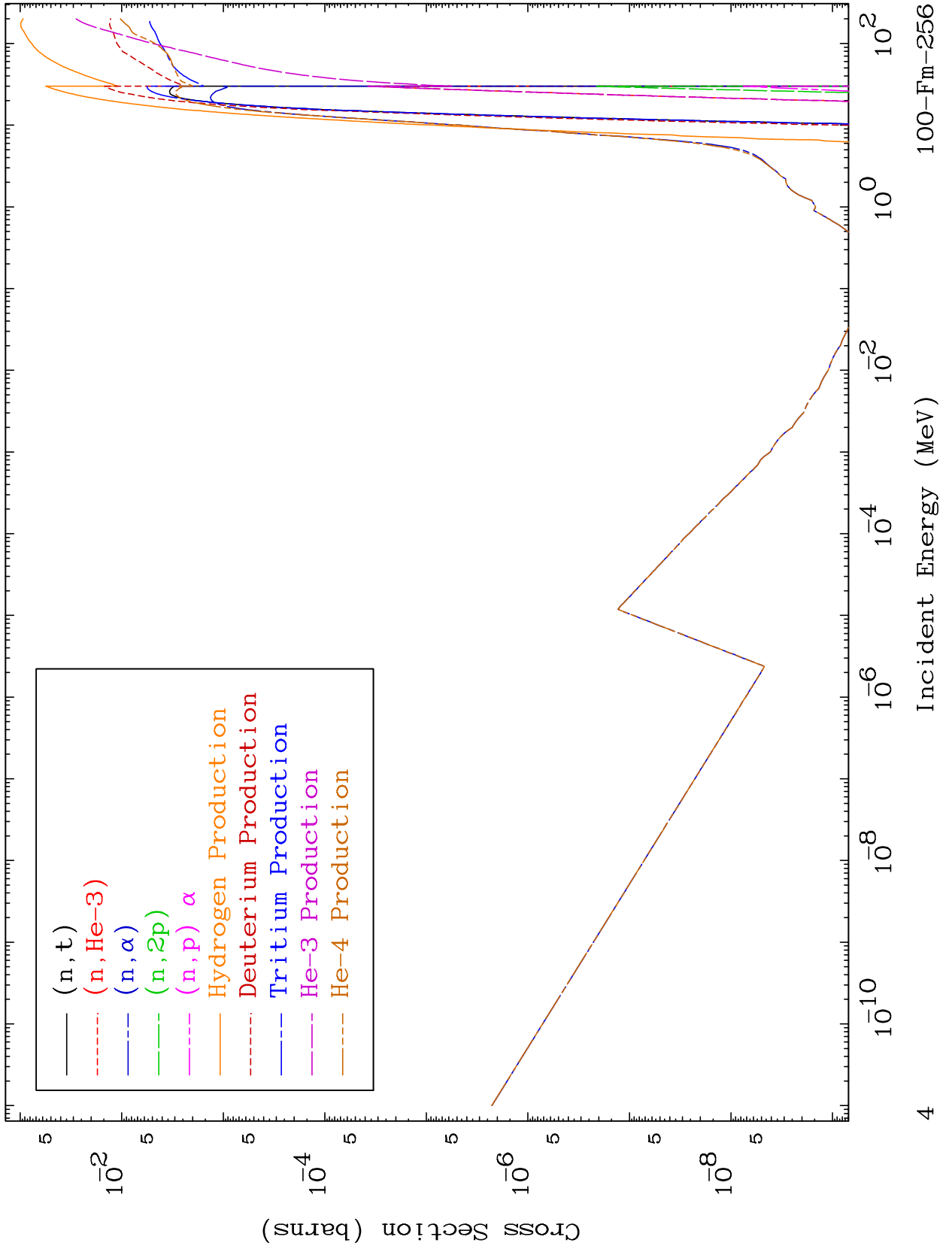
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MAT 9937

Neutron Absorption
293 Kelvin Cross Sections

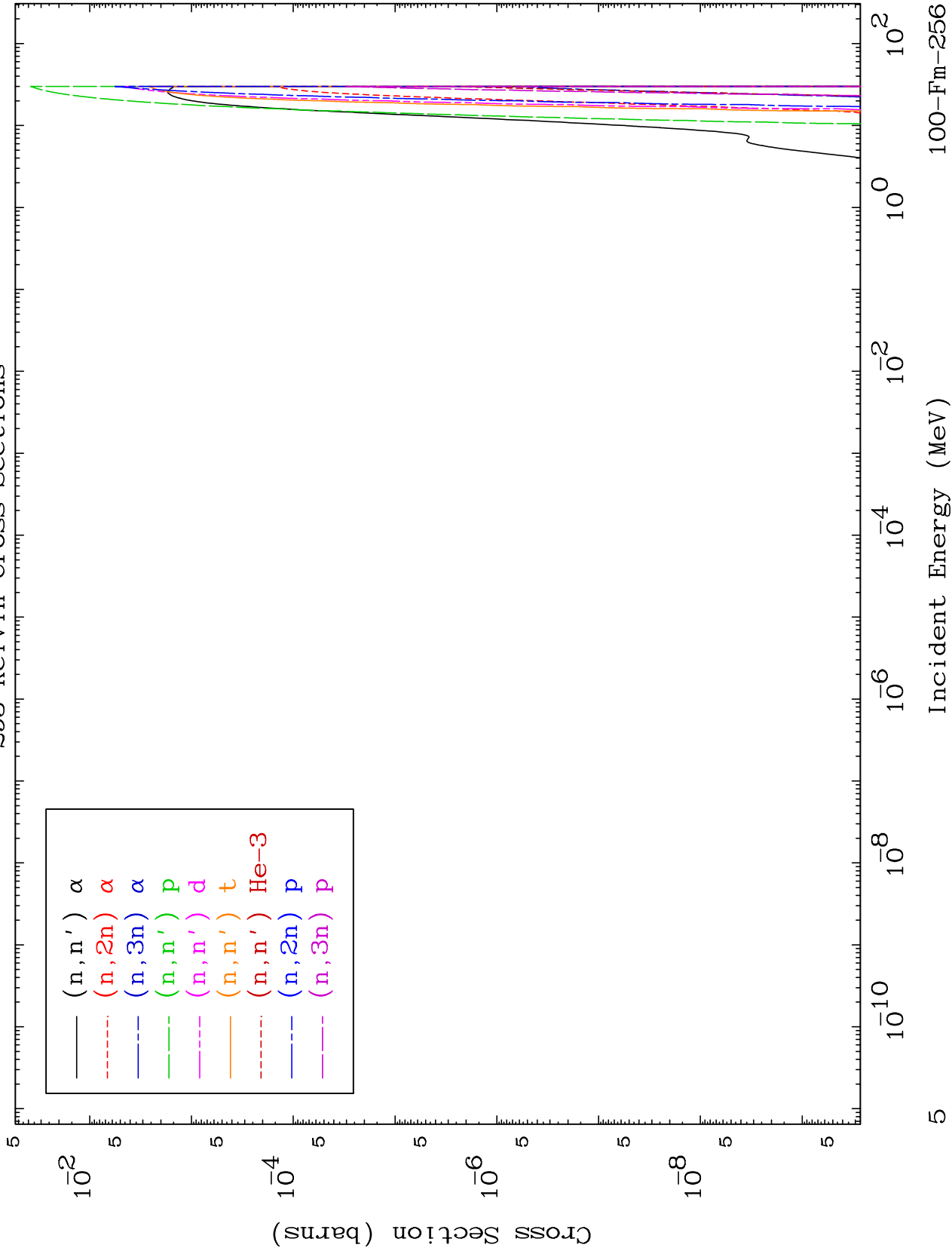
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MAT 9937

Charged Particle
293 Kelvin Cross Sections

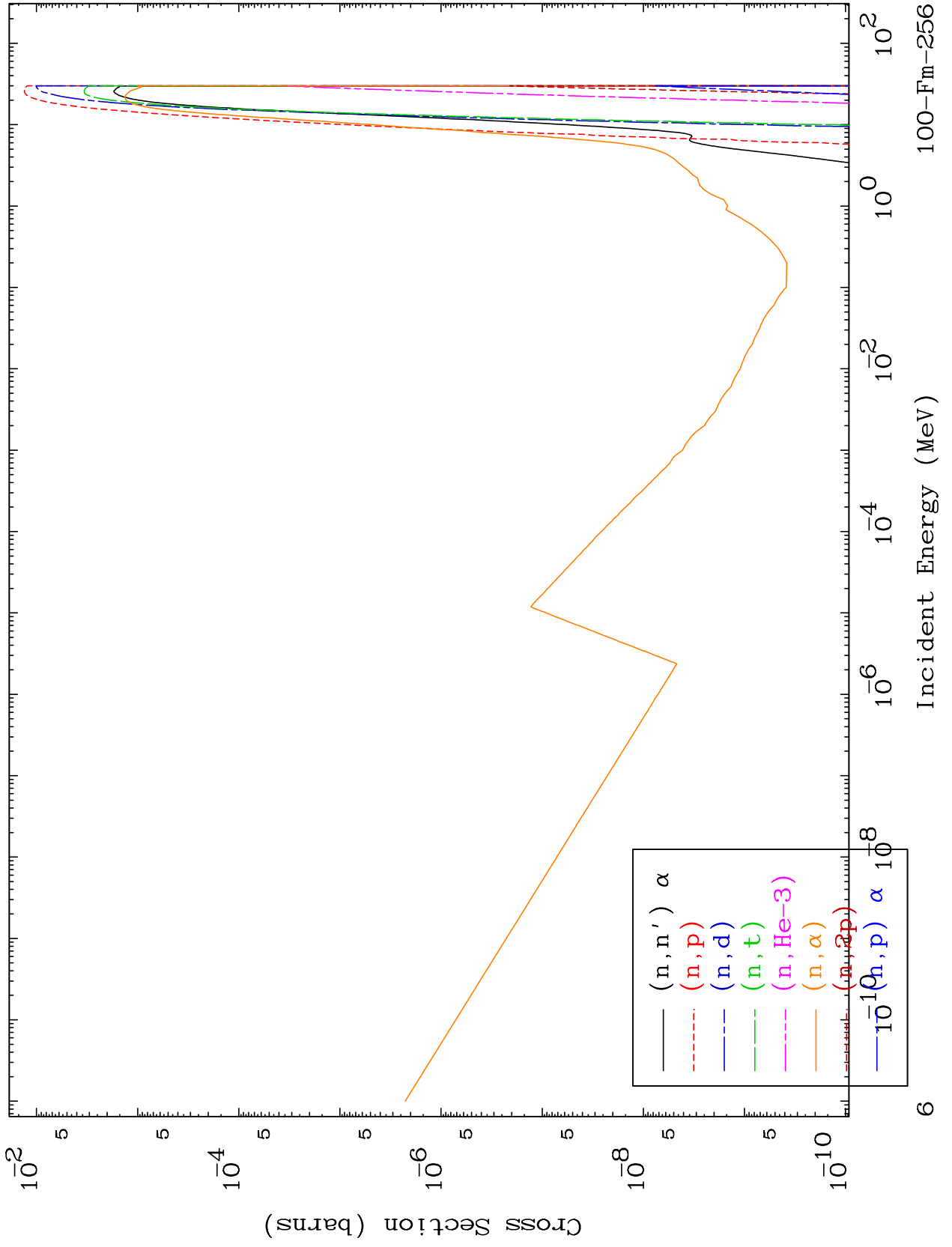
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MAT 9937

Charged Particle
293 Kelvin Cross Sections

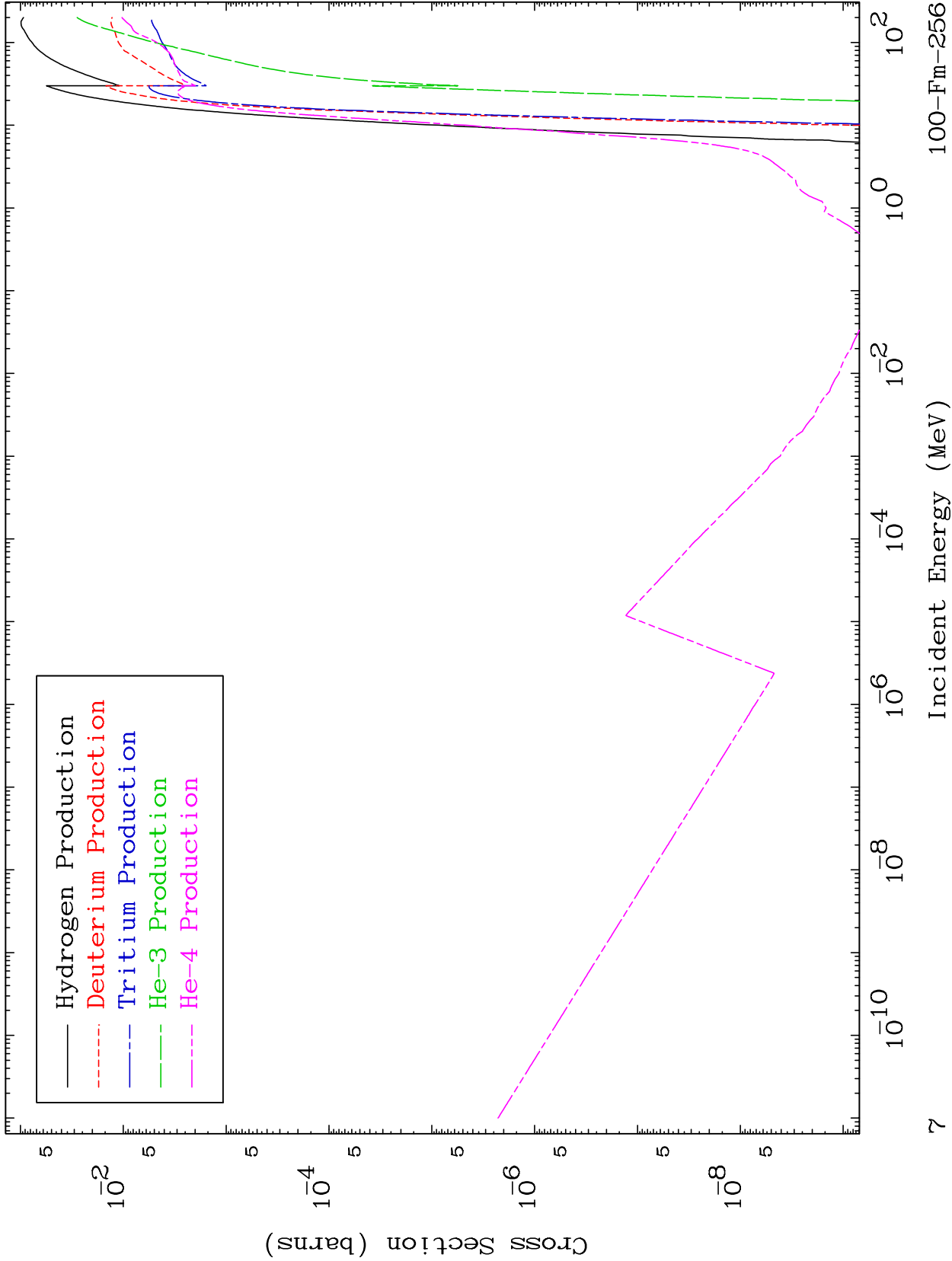
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MAT 9937

Particle Production
293 Kelvin Cross Sections

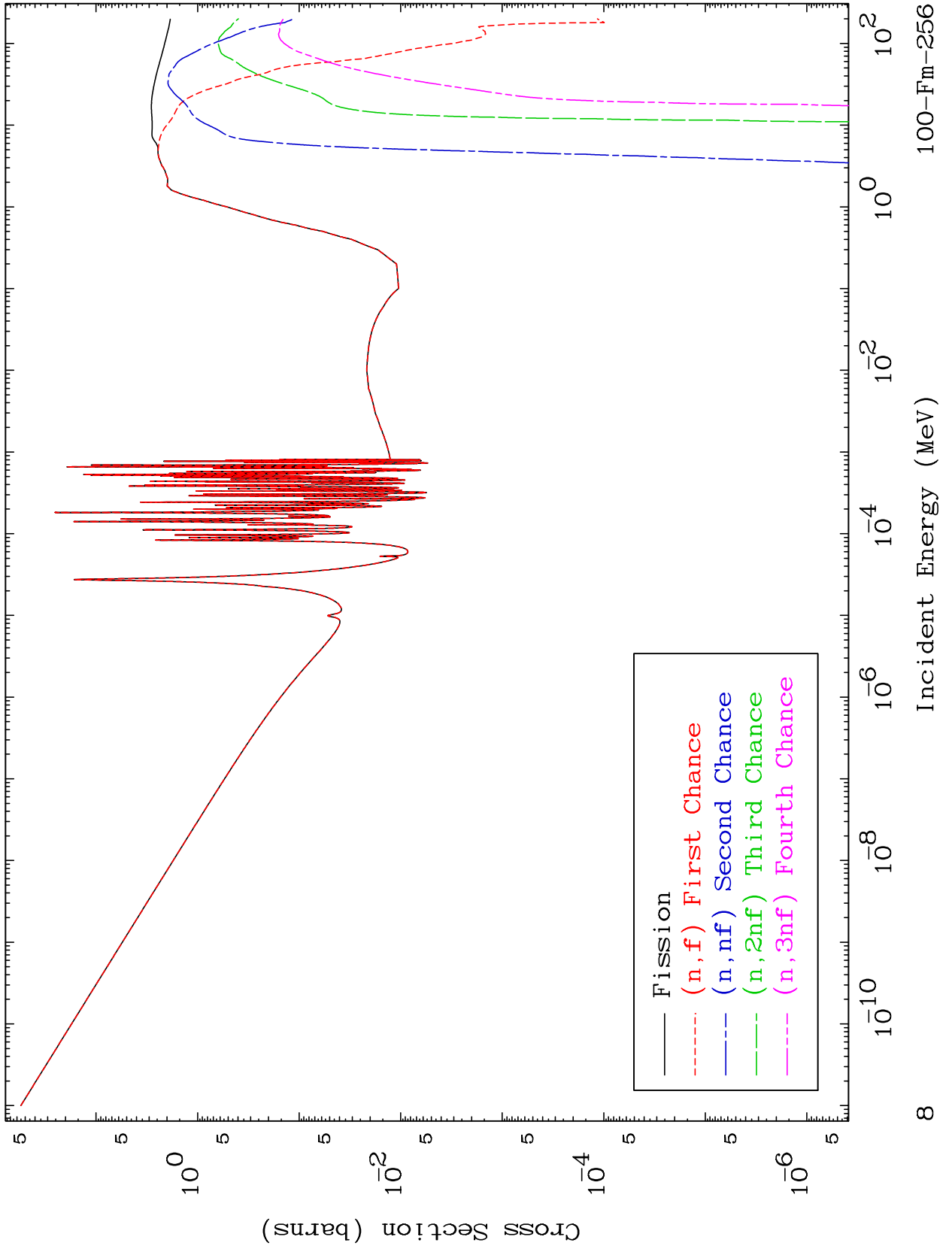
100-Fm-256



MAT 9937

Fission
293 Kelvin Cross Sections

100-Fm-256

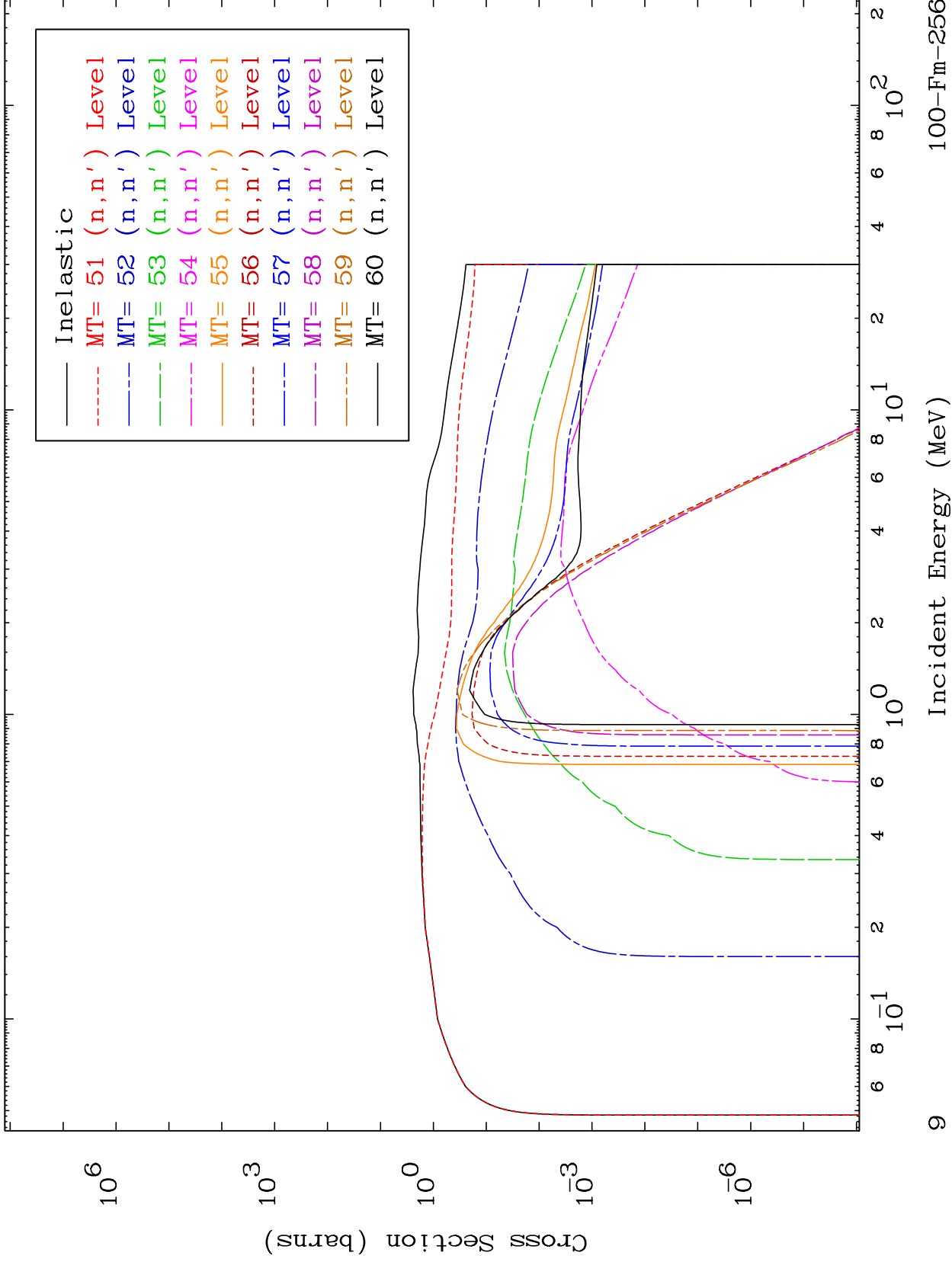


MAT 9937

(n,n') Levels

100-Fm-256

293 Kelvin Cross Sections

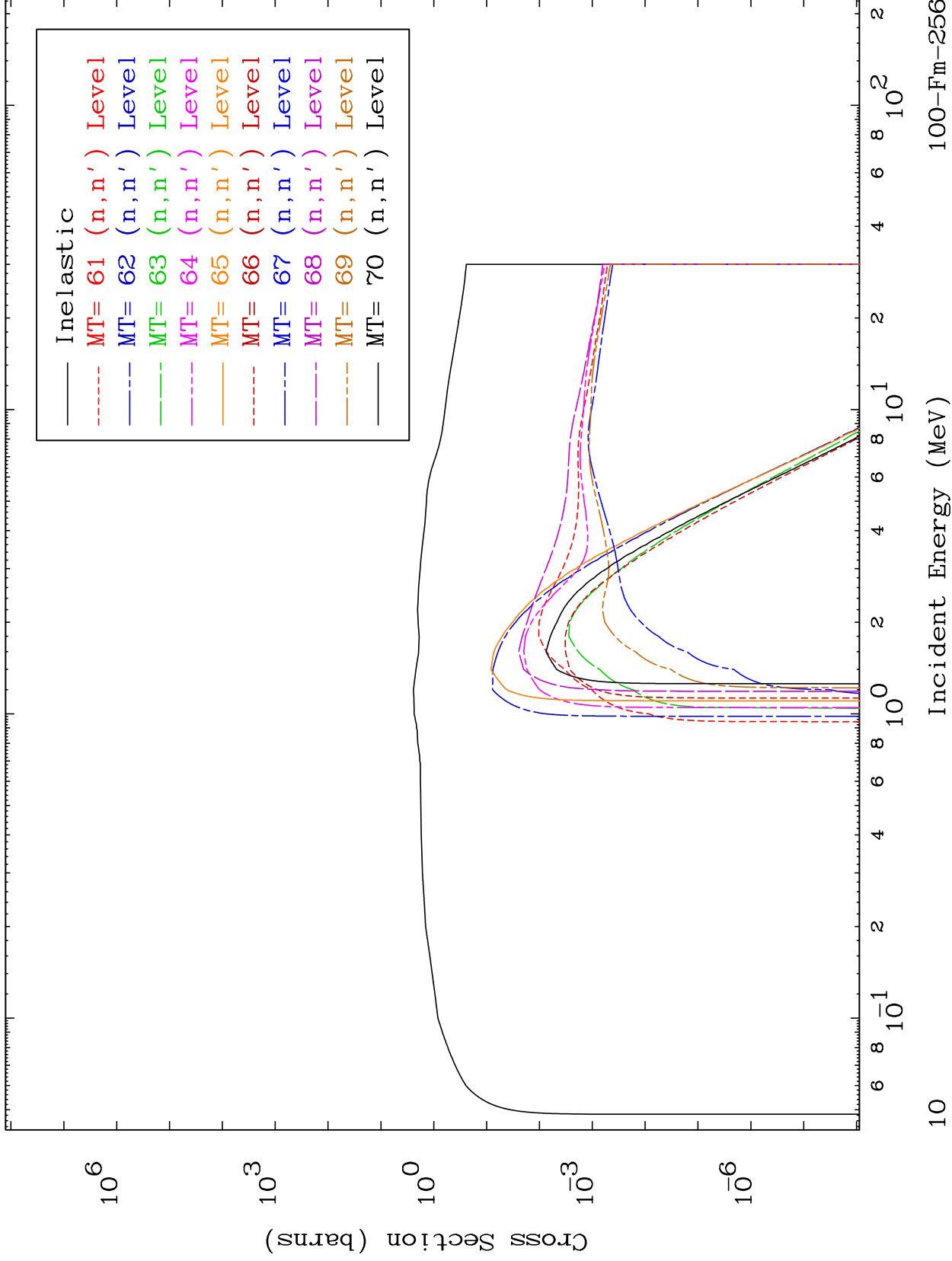


MAT 9937

(n,n') Levels

100-Fm-256

293 Kelvin Cross Sections



10

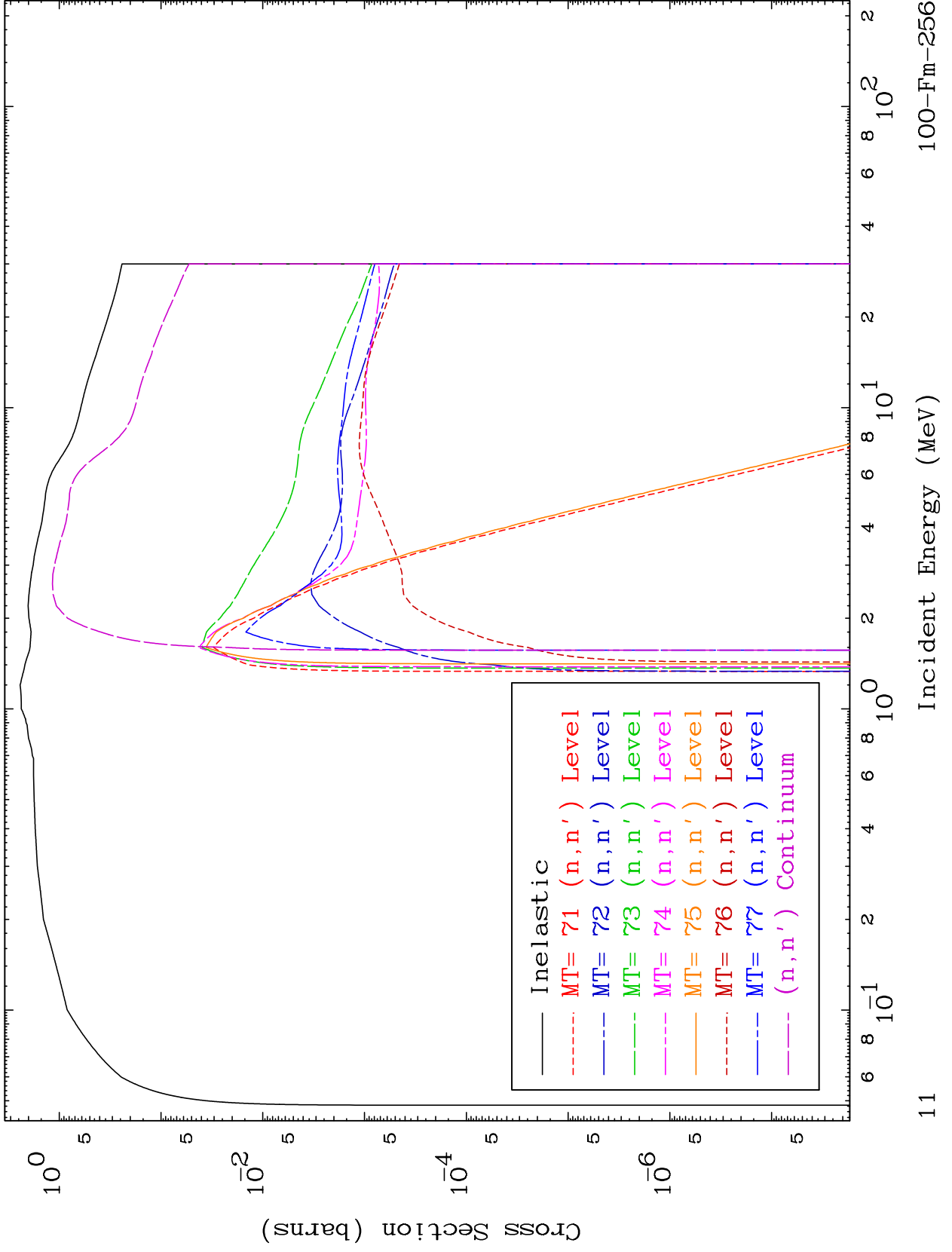
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,n') Levels
293 Kelvin Cross Sections

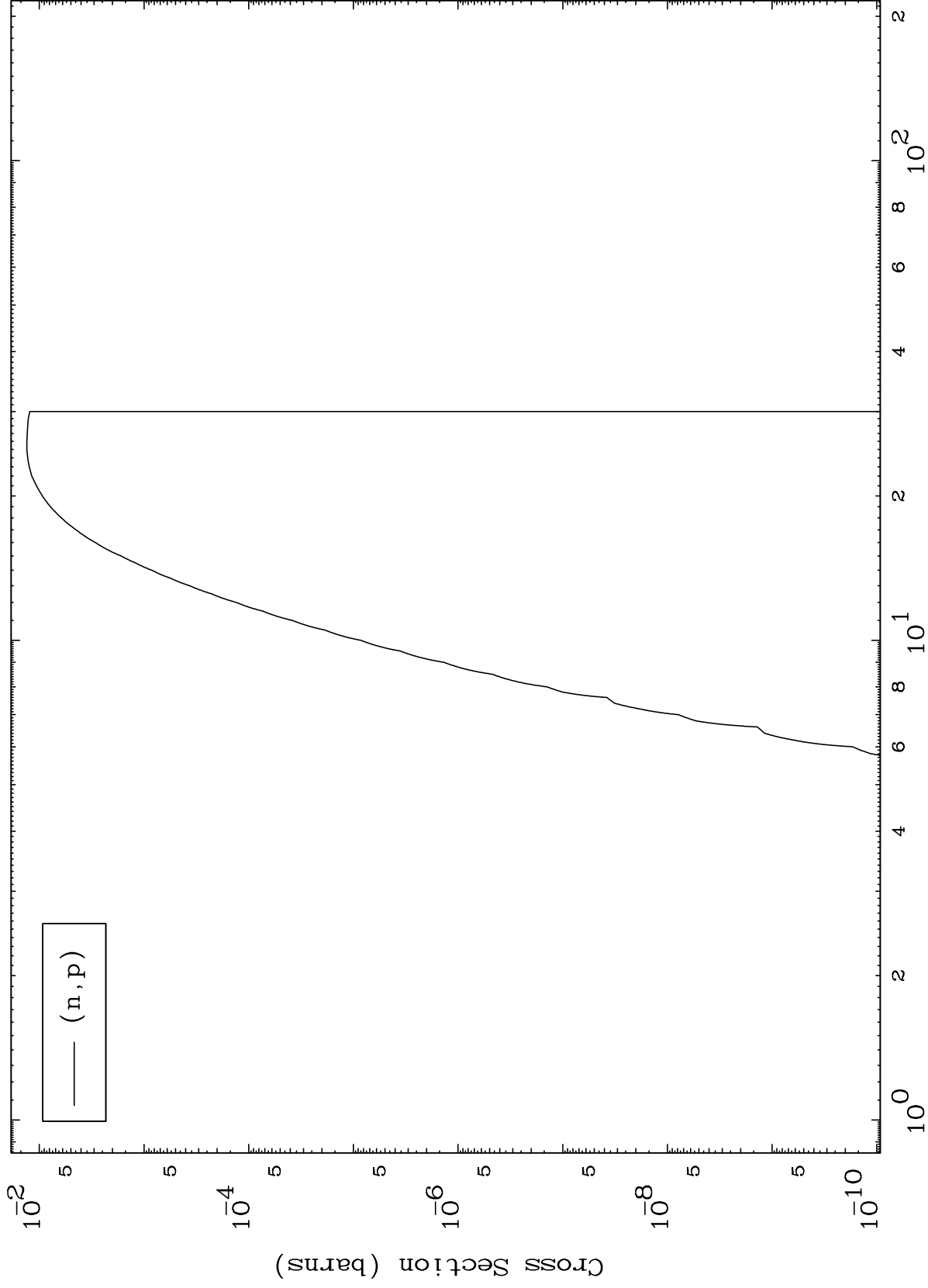
100-Fm-256



MAT 9937

(n,p) Levels
293 Kelvin Cross Sections

100-Fm-256



Incident Energy (MeV)

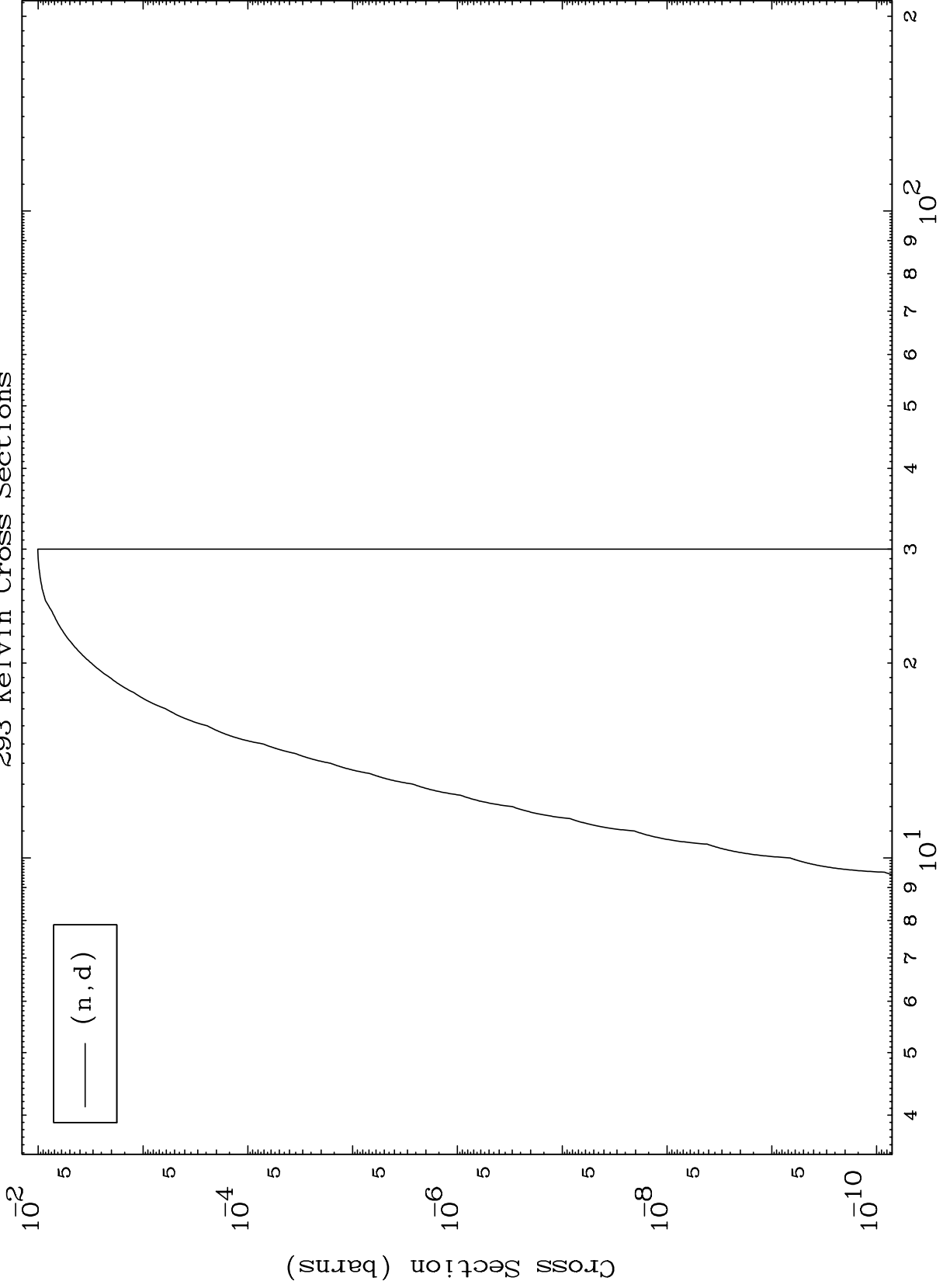
100-Fm-256

12

MAT 9937

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-256



13

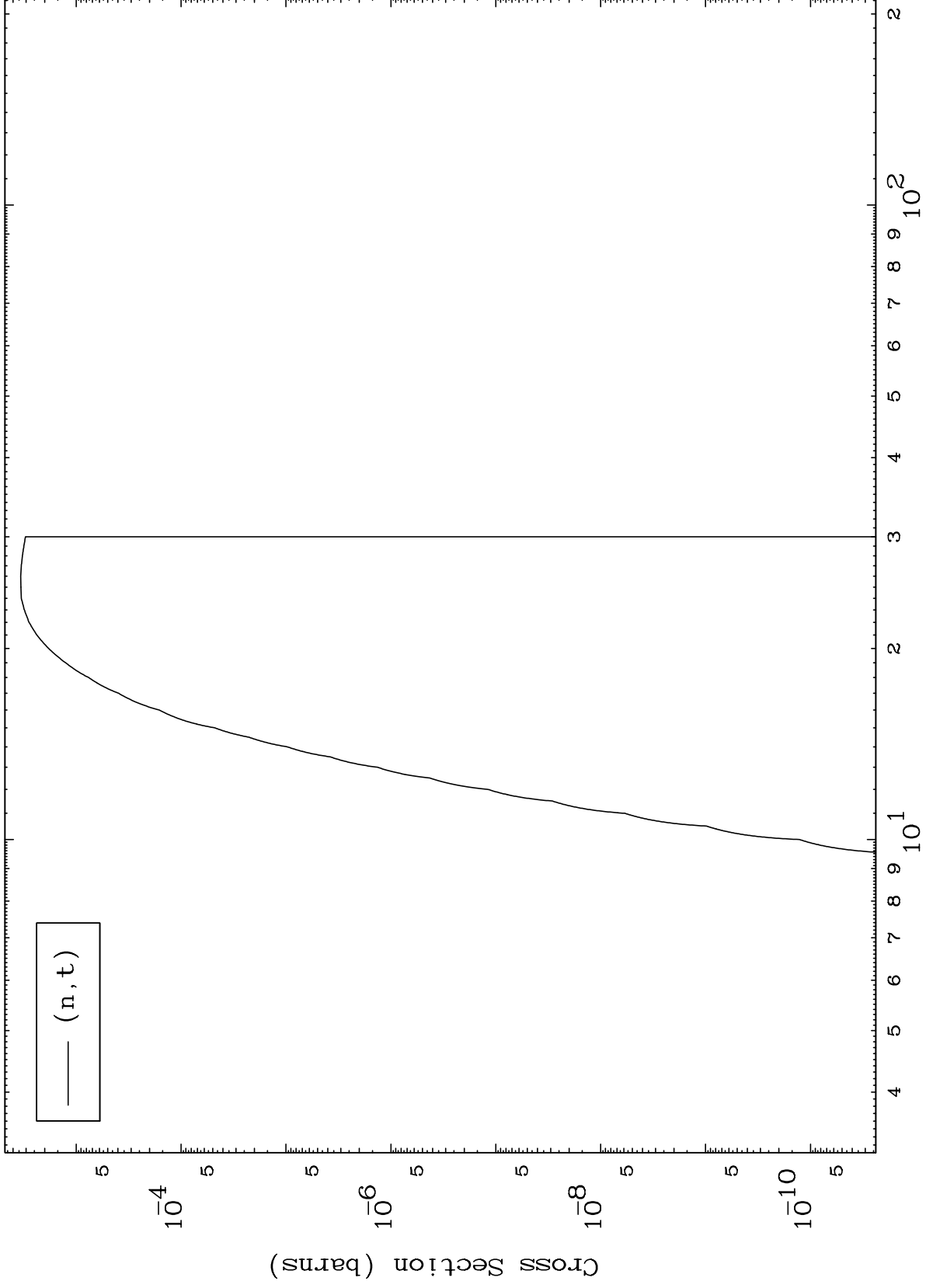
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-256



14

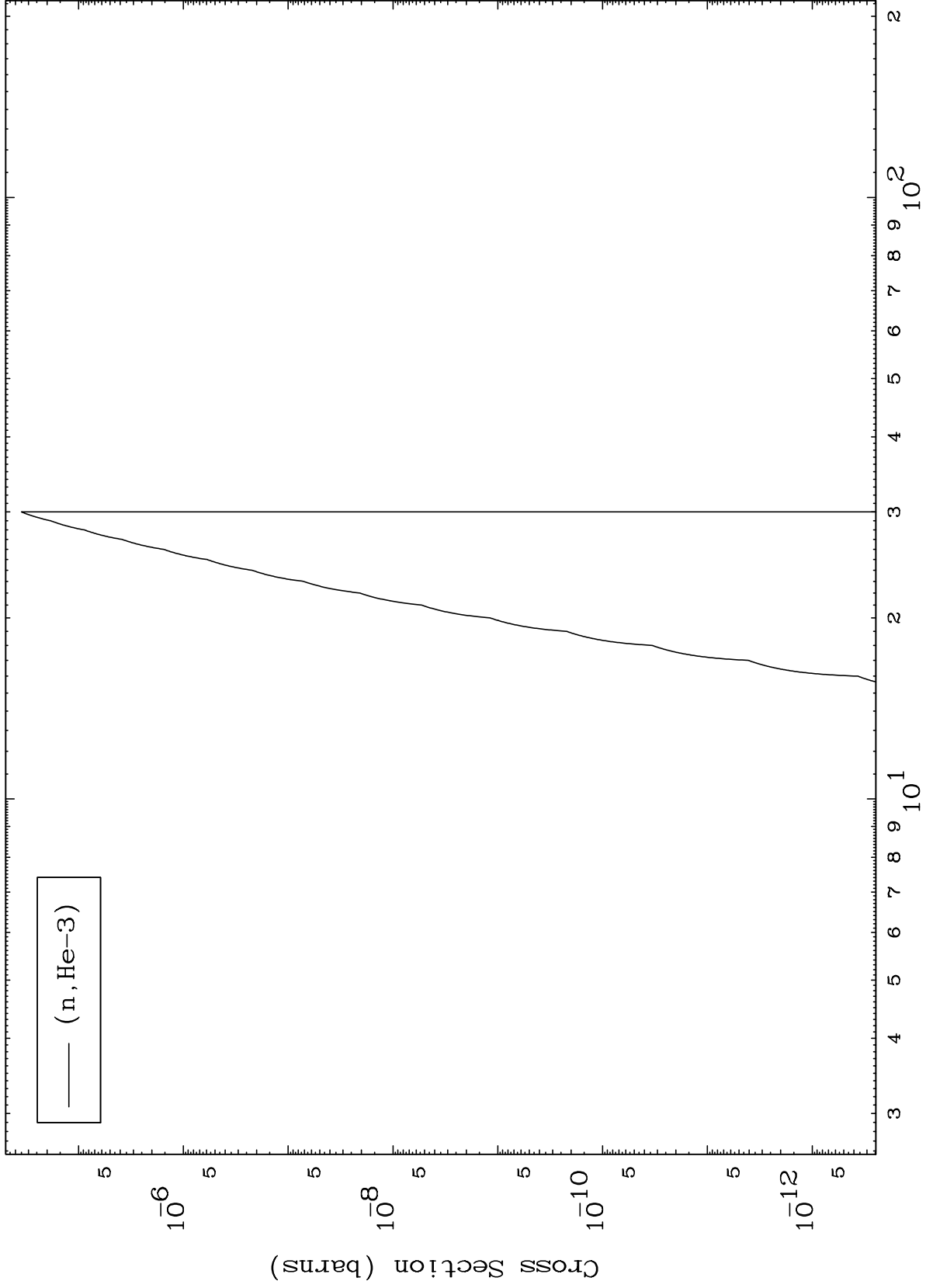
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-256



15

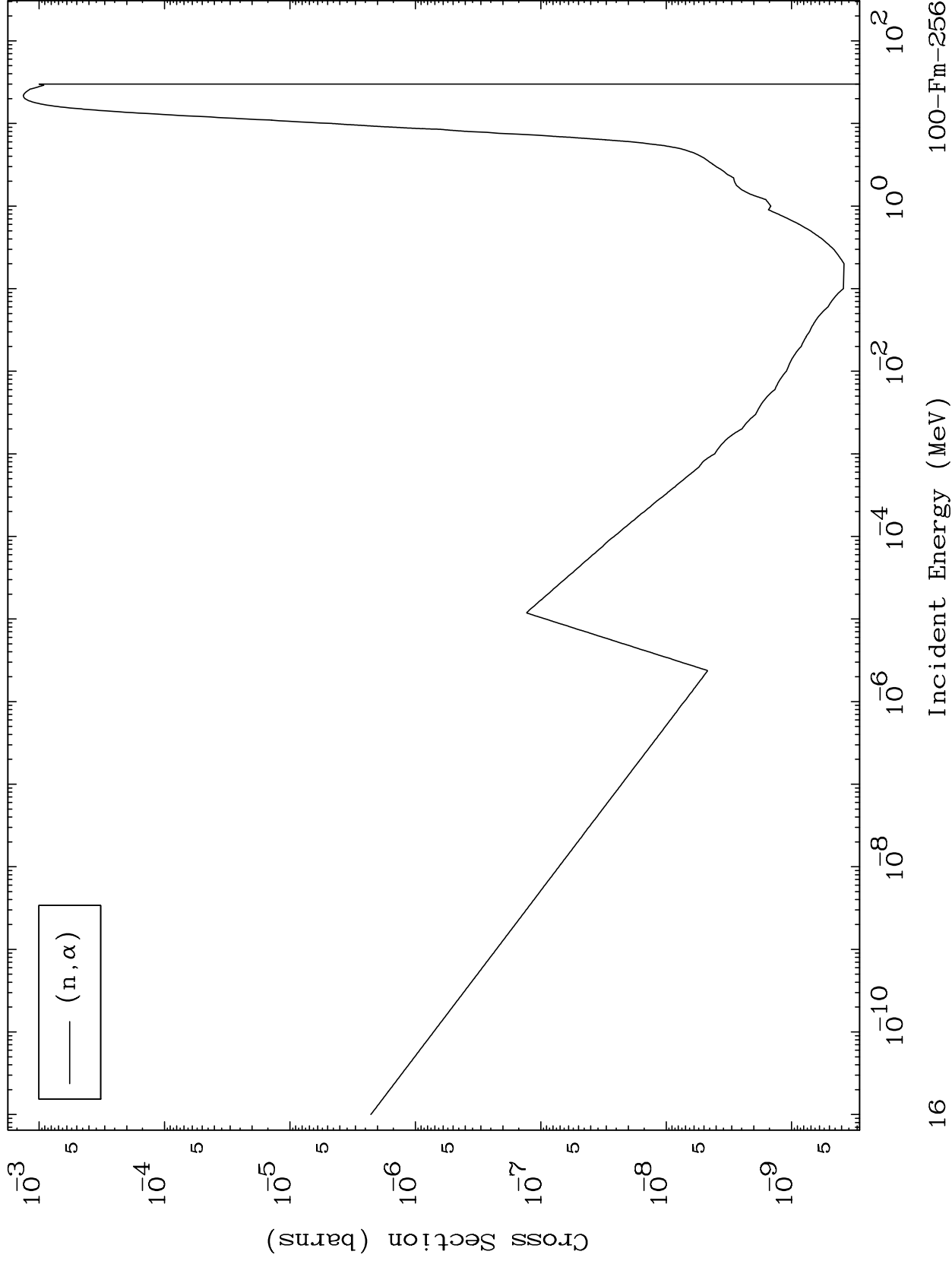
Incident Energy (MeV)

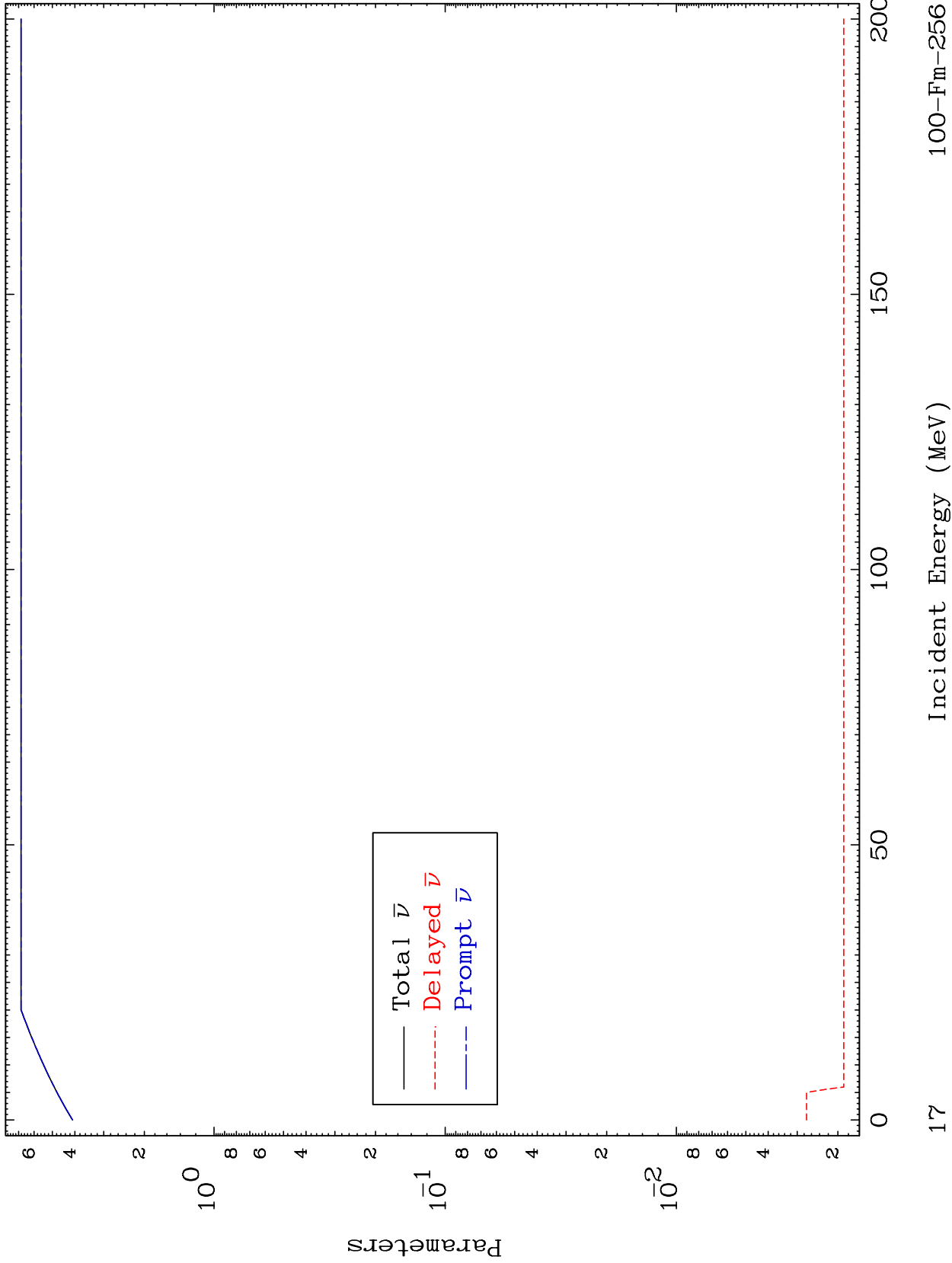
100-Fm-256

MAT 9937

(n, α) Levels
293 Kelvin Cross Sections

100-Fm-256

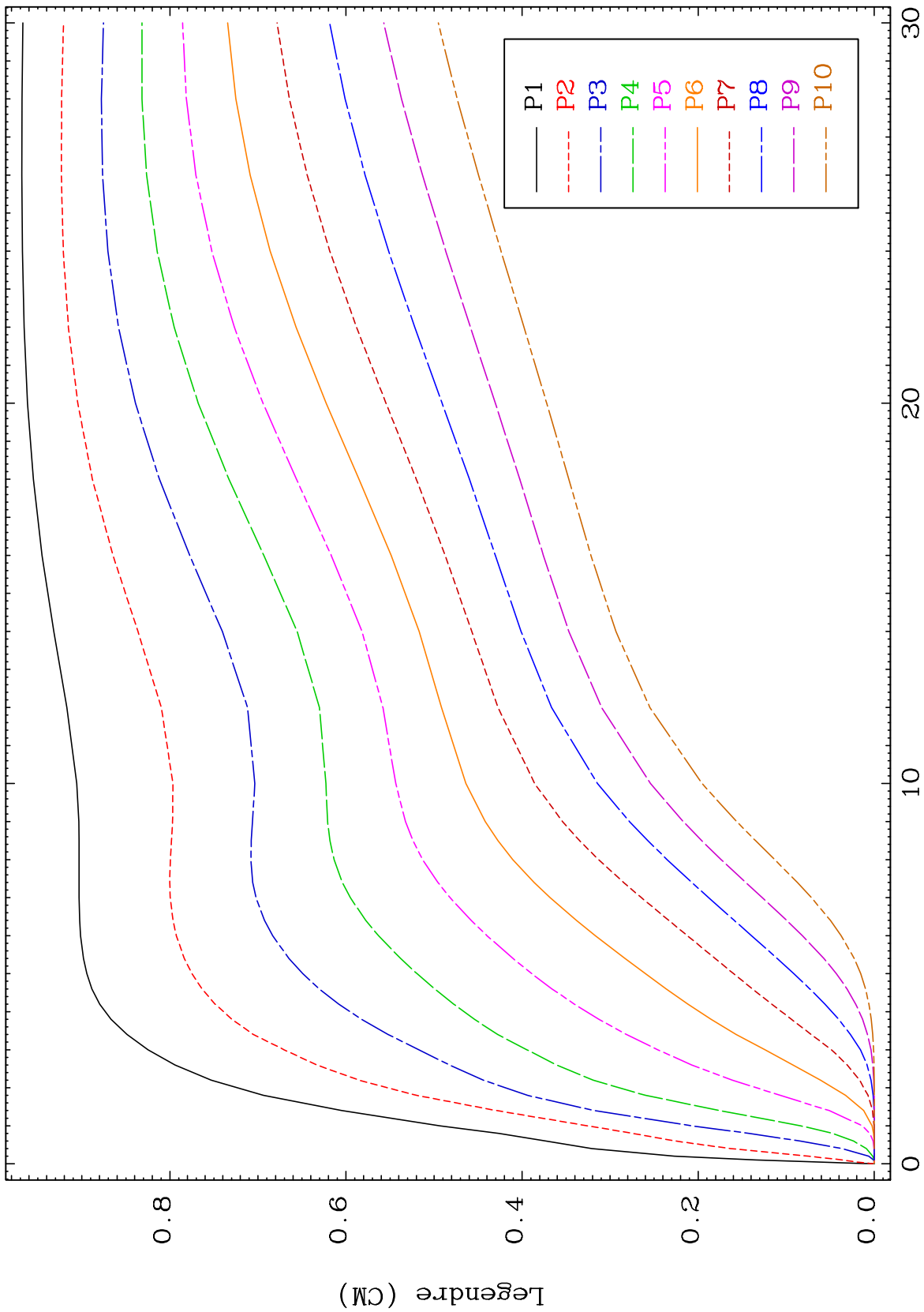




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Elastic Legendre Coefficients

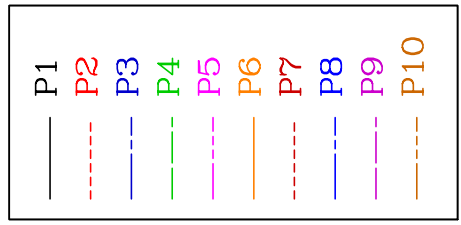
100-Fm-256



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Incident Energy (MeV)

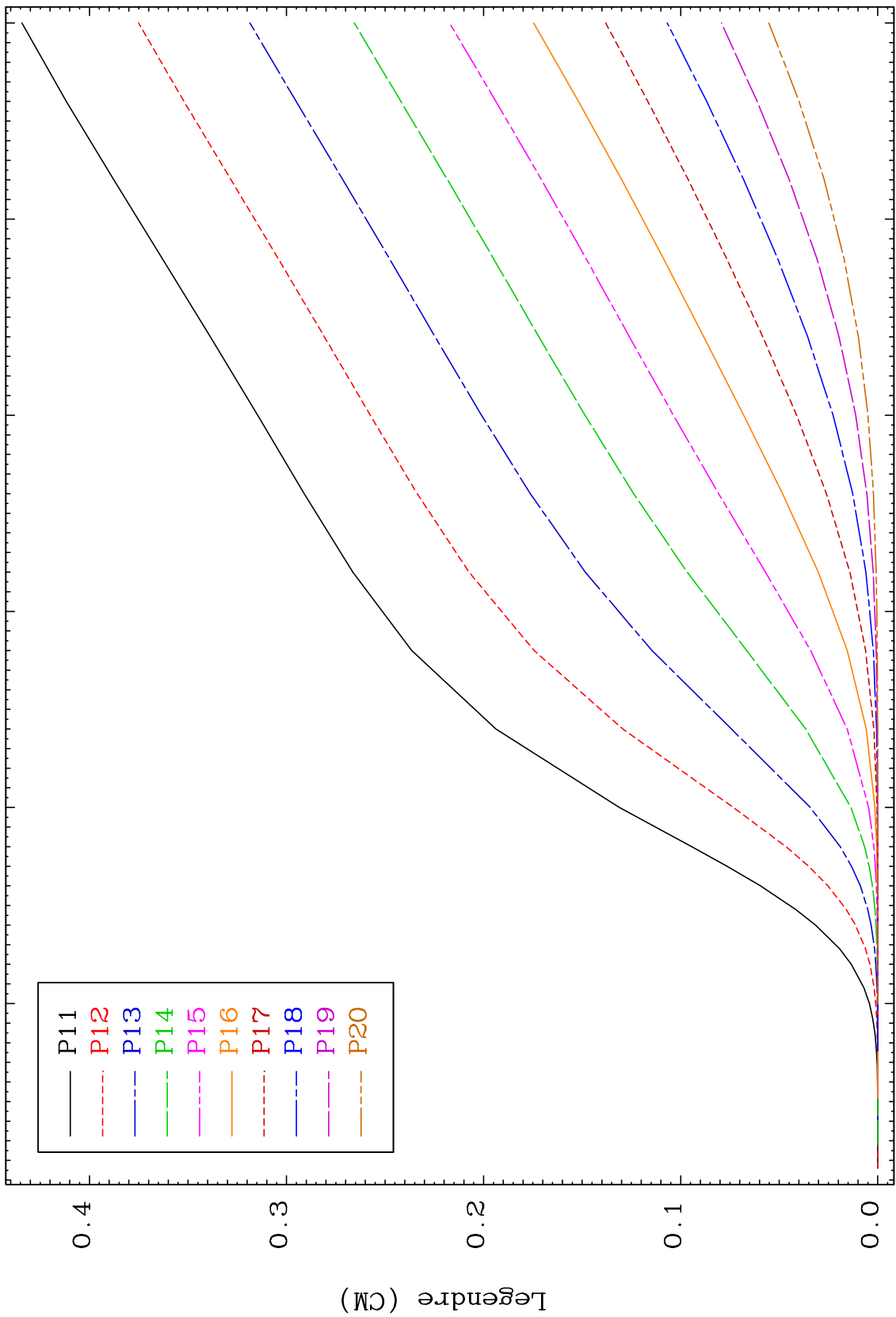
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Elastic Legendre Coefficients

100-Fm-256



19

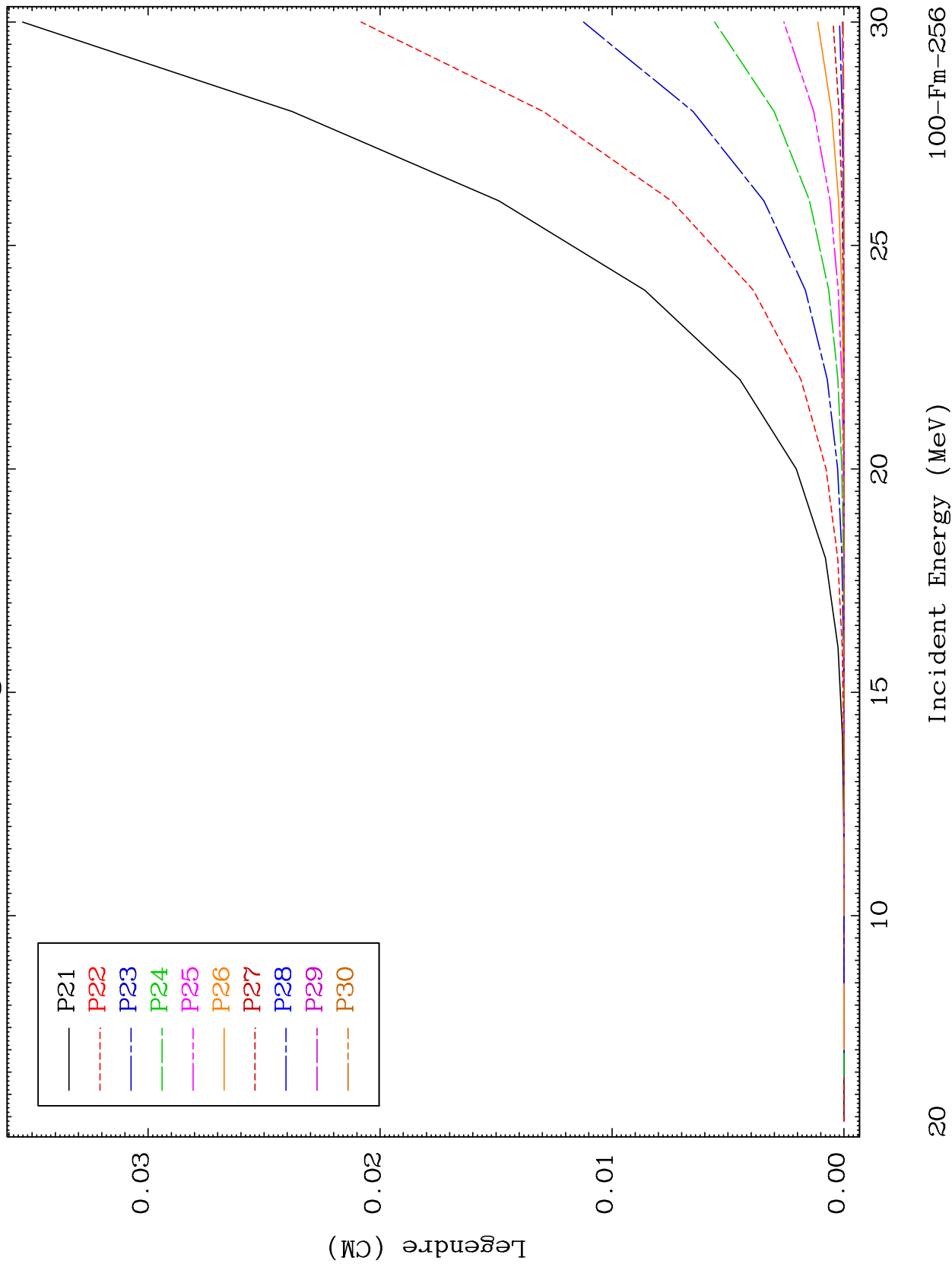
100-Fm-256

Incident Energy (MeV)

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Elastic
Legendre Coefficients

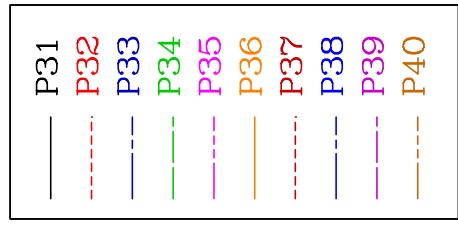
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MAT 9937

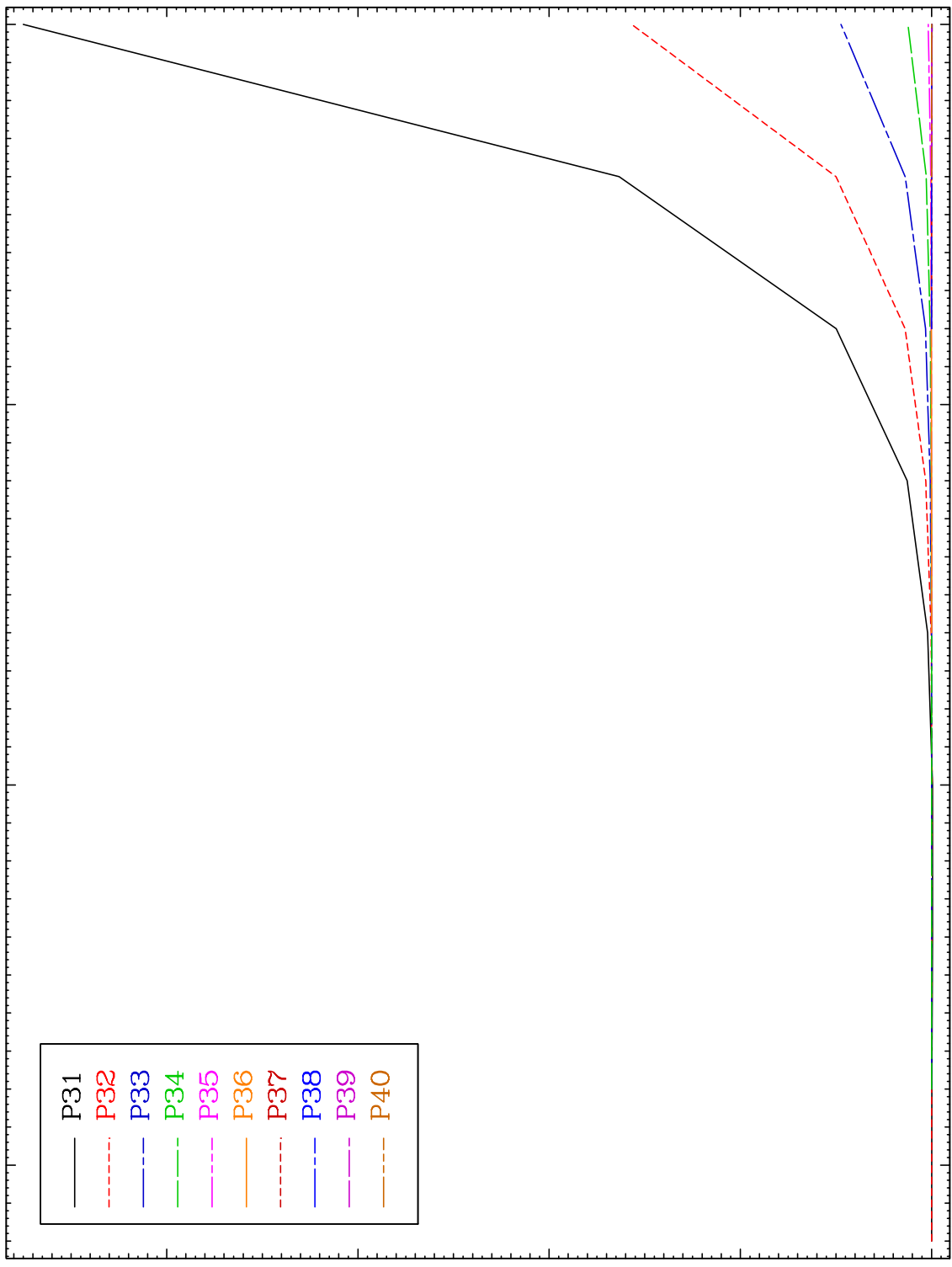
Elastic
Legendre Coefficients

100-Fm-256



$\times 10^{-6}$

Legendre (CM)



21

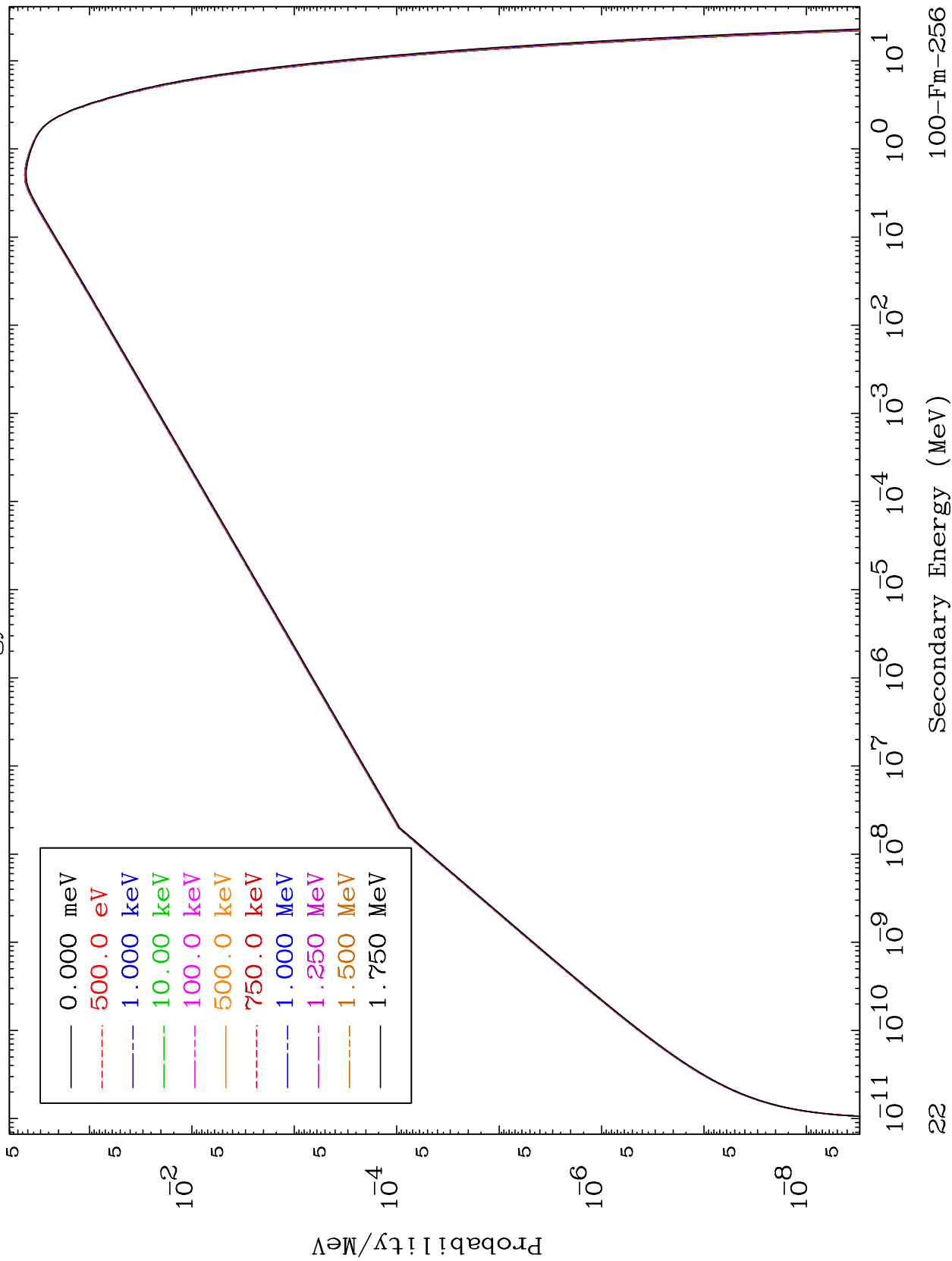
Incident Energy (MeV)

100-Fm-256

MAT 9937

Fission Energy Distributions

100-Fm-256



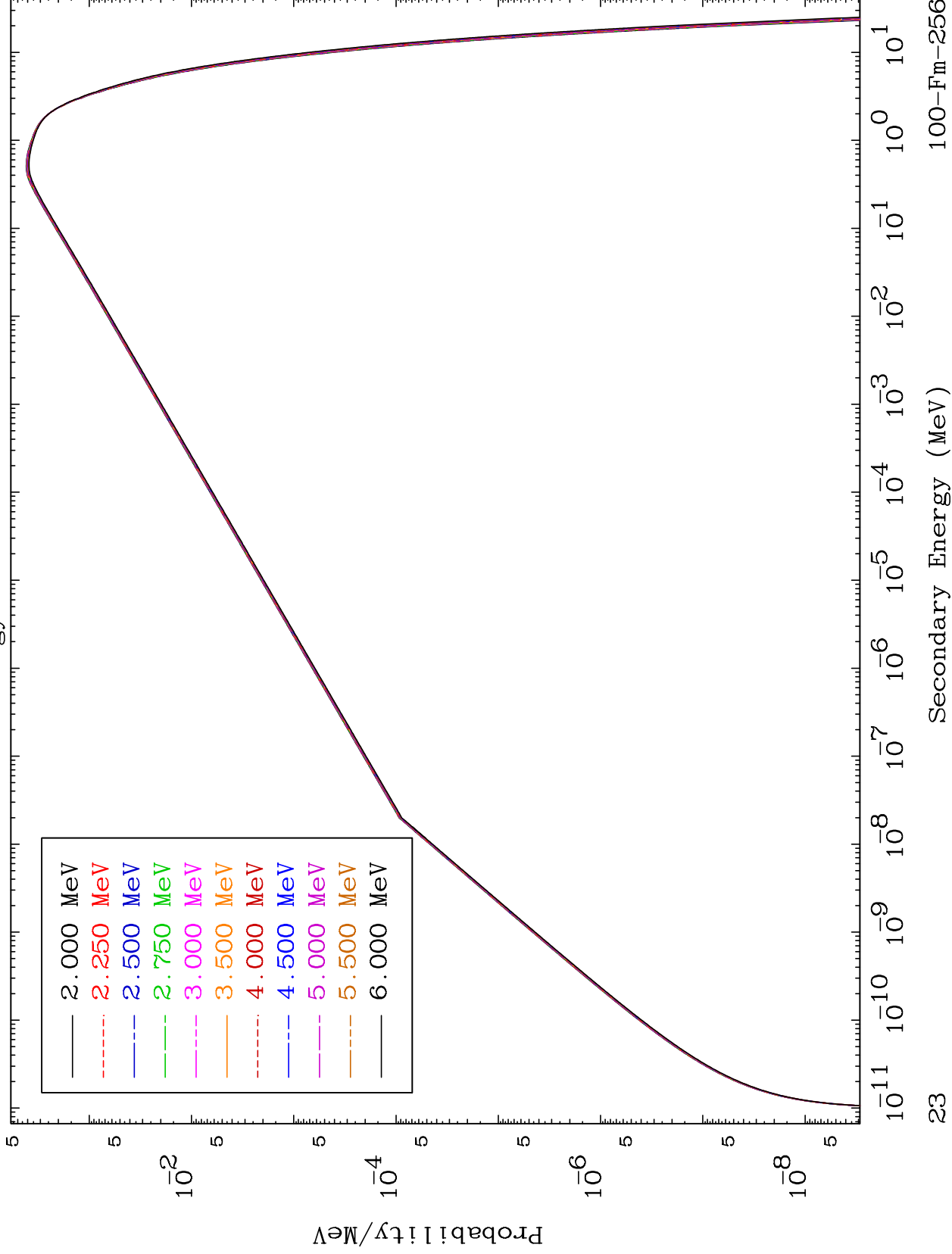
100-Fm-256

Secondary Energy (MeV)

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Fission Energy Distributions

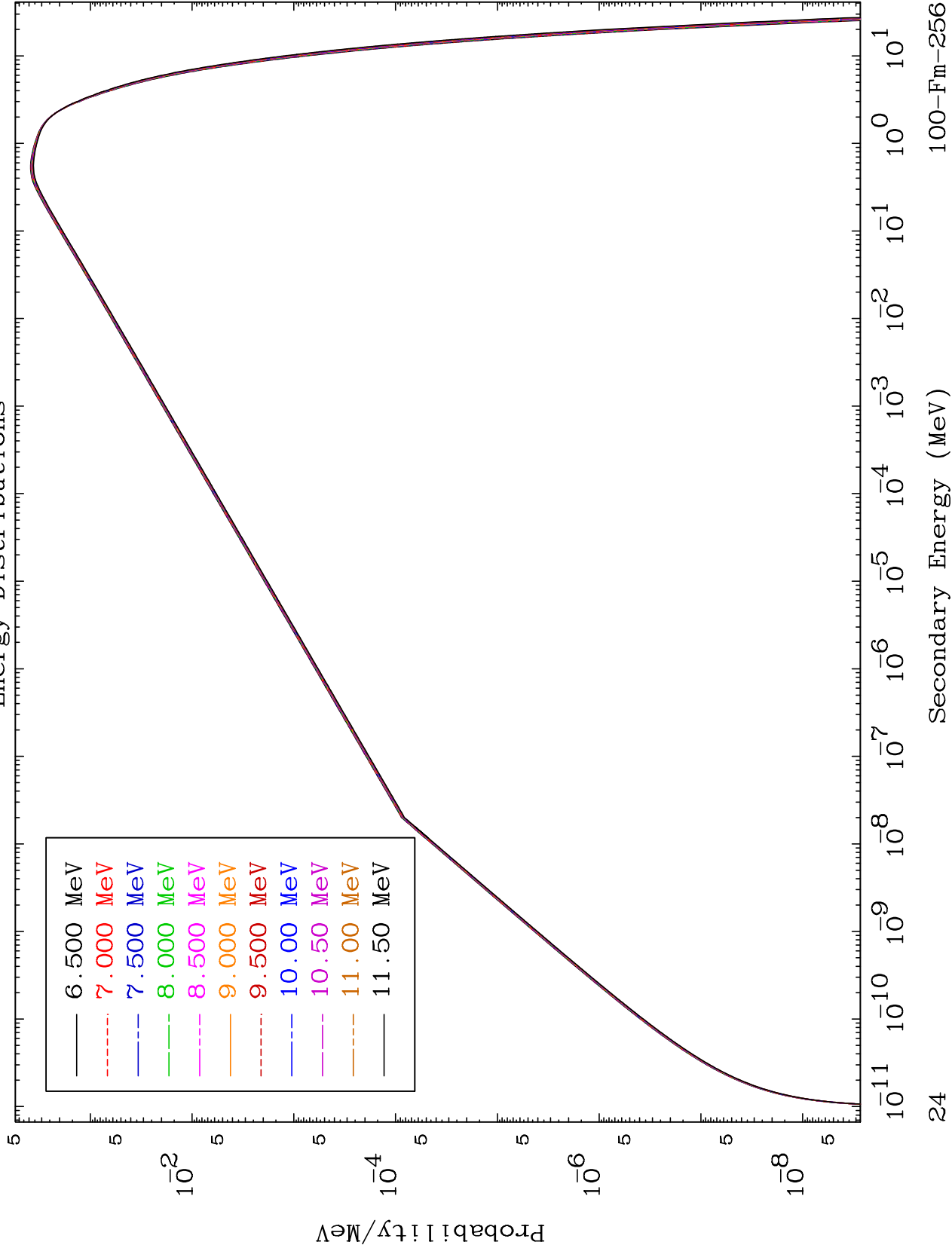
100-Fm-256



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Fission Energy Distributions

100-Fm-256



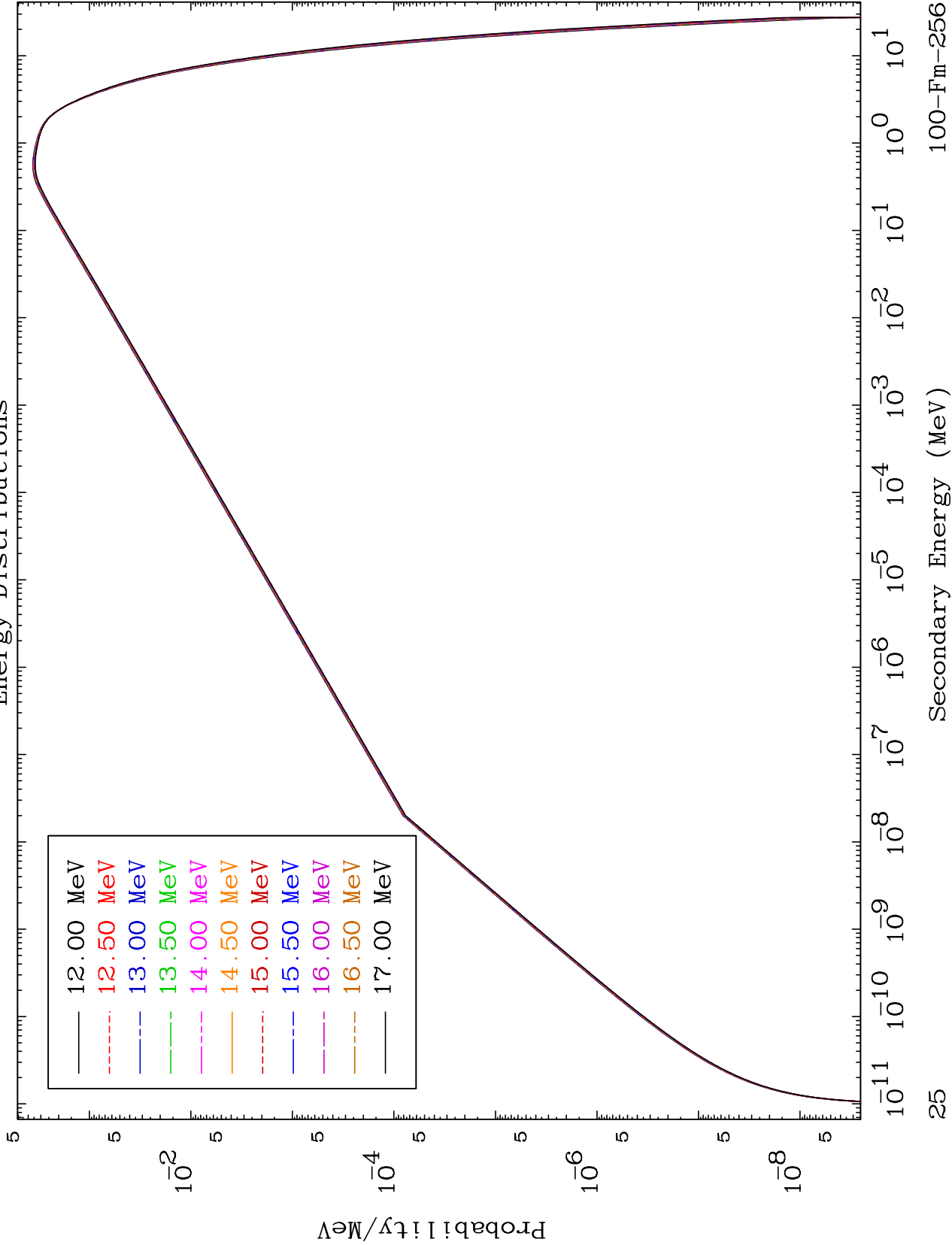
100-Fm-256

Secondary Energy (MeV)

MAT 9937

Fission Energy Distributions

100-Fm-256

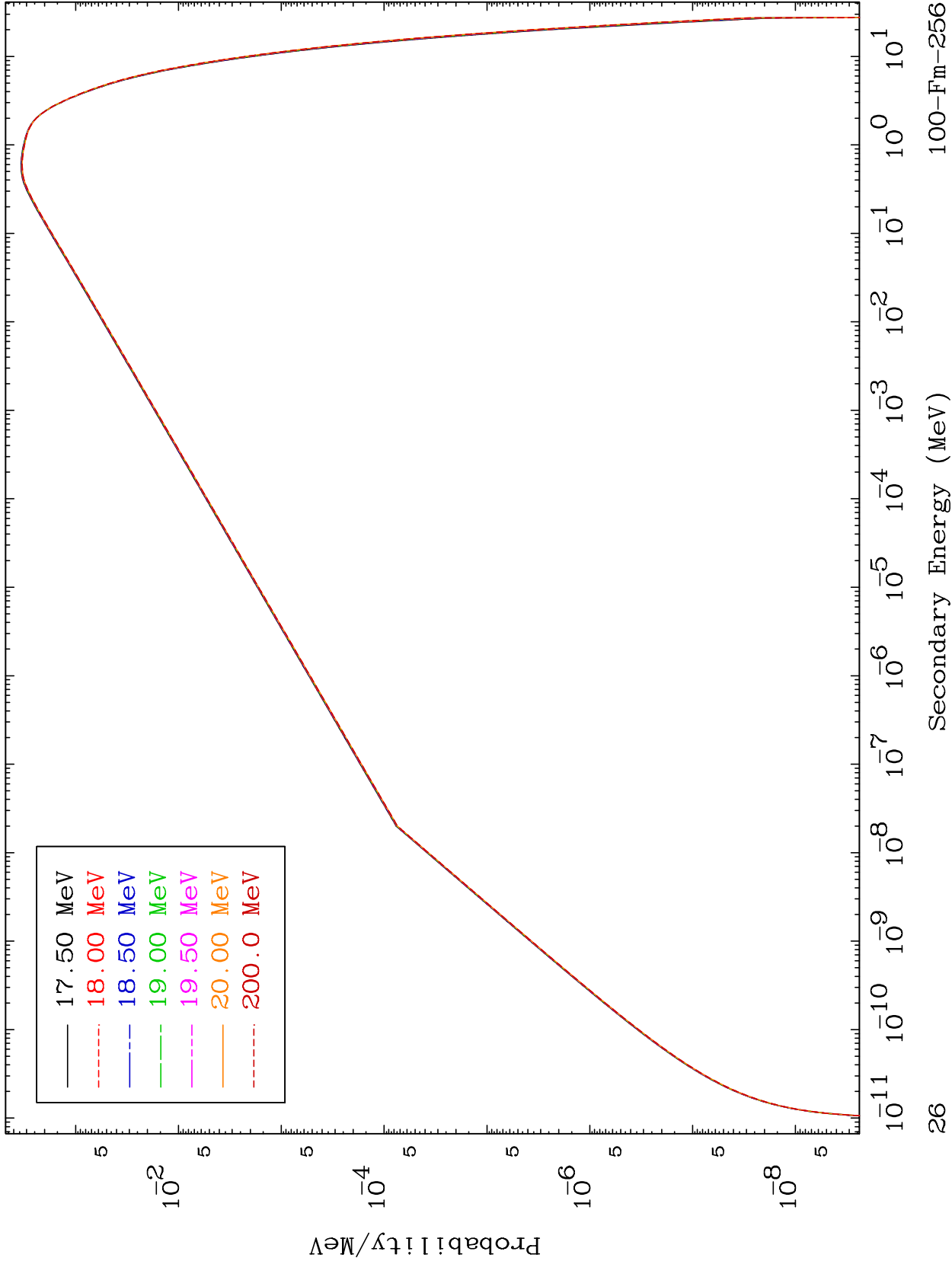


100-Fm-256

Secondary Energy (MeV)

25

Fission Energy Distributions

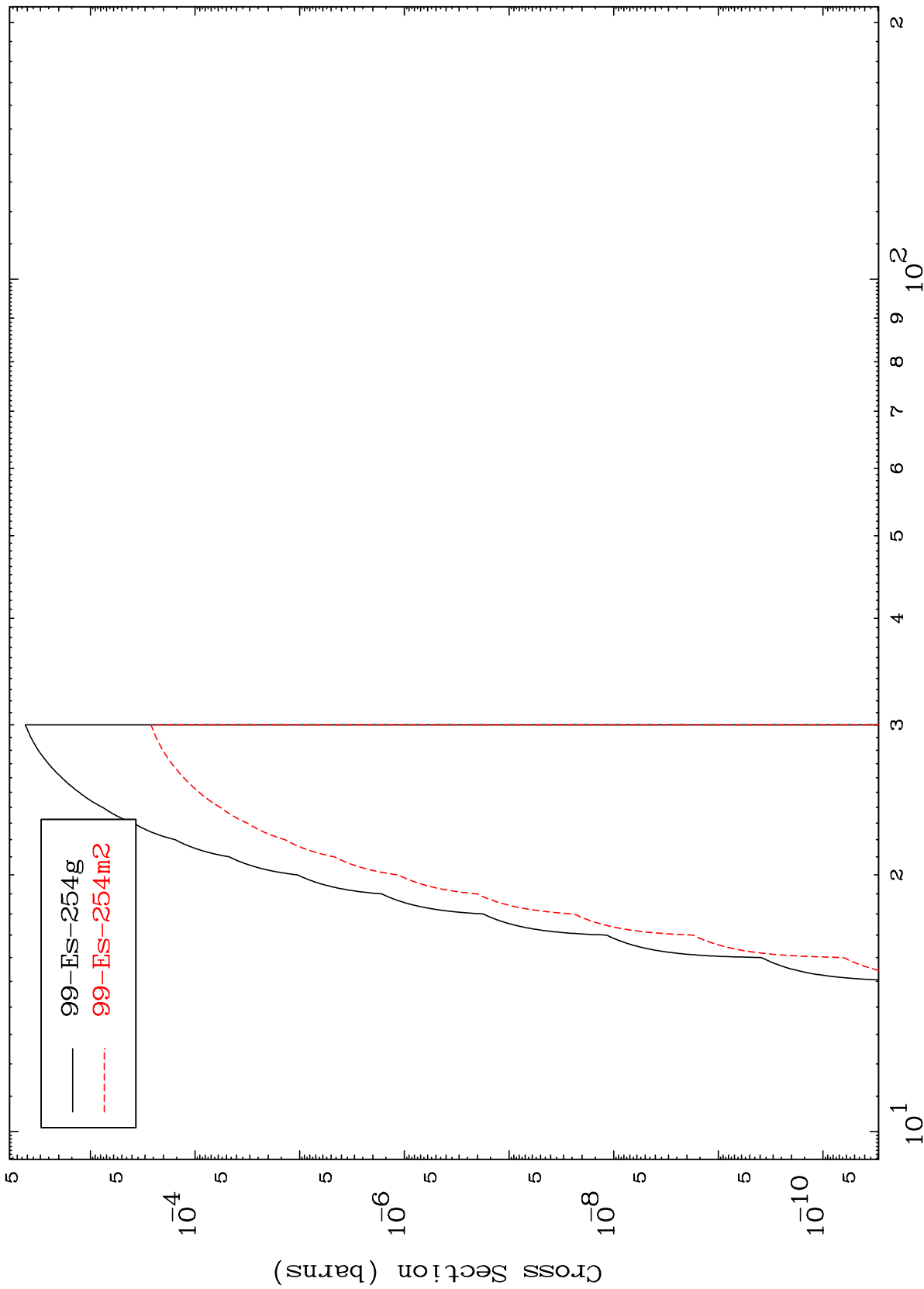


MAT 9937

(n,n') d

100-Fm-256

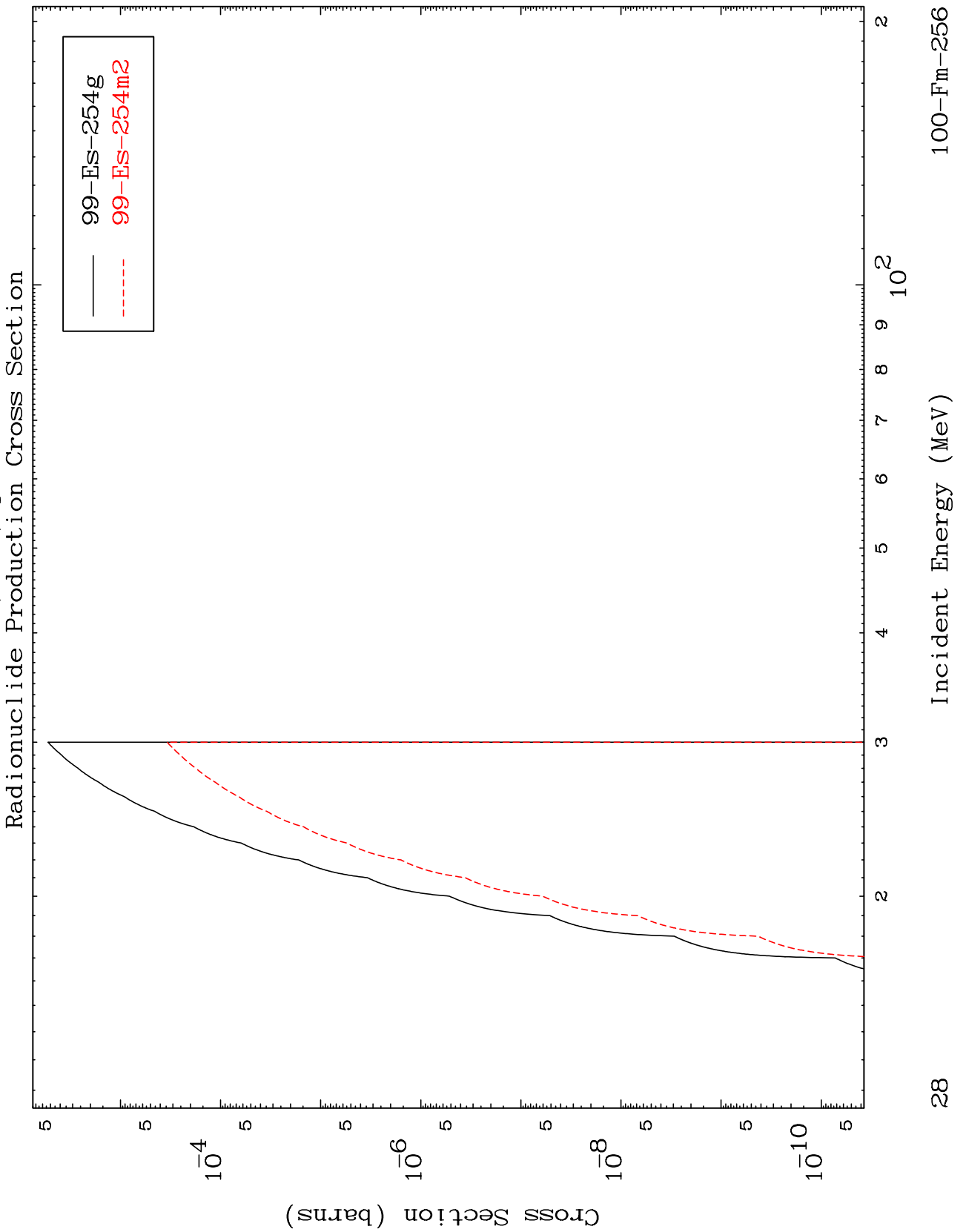
Radionuclide Production Cross Section



27

Incident Energy (MeV)

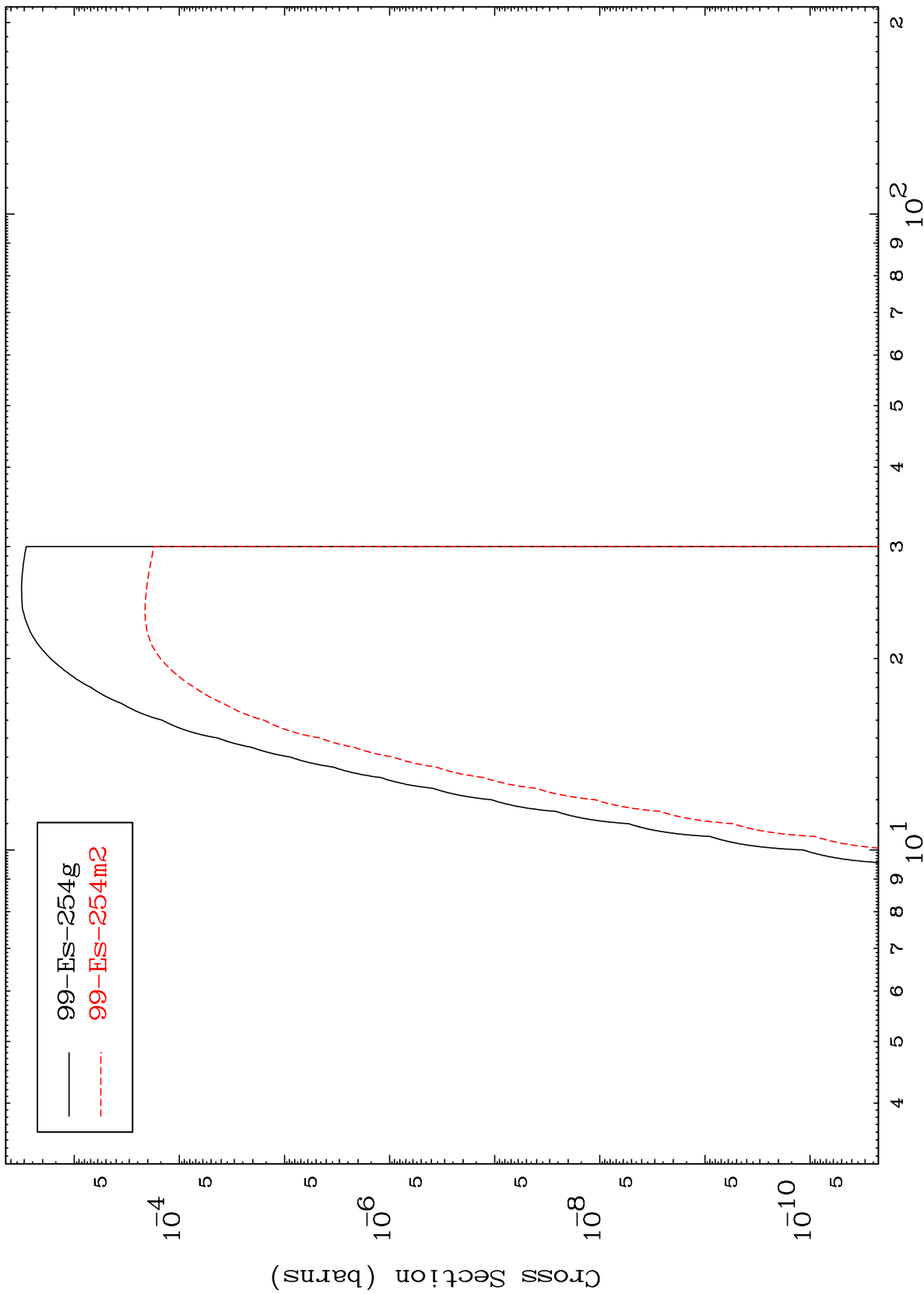
100-Fm-256



MAT 9937

100-Fm-256

(n, t)
Radionuclide Production Cross Section



29

Incident Energy (MeV)

100-Fm-256