

Program EVALPLOT  
(Version 2021-1)

by

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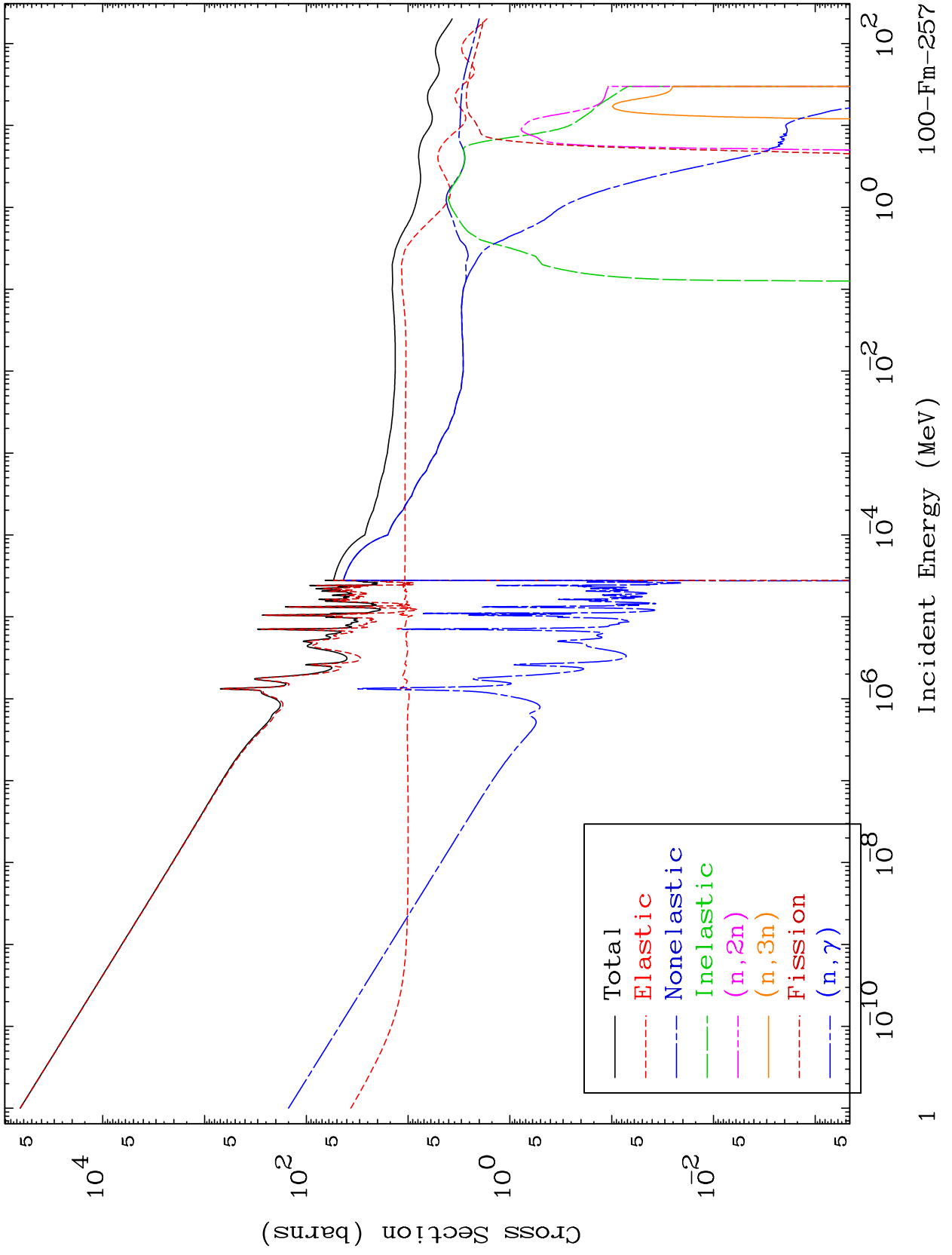
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9938

Neutron Major  
293 Kelvin Cross Sections

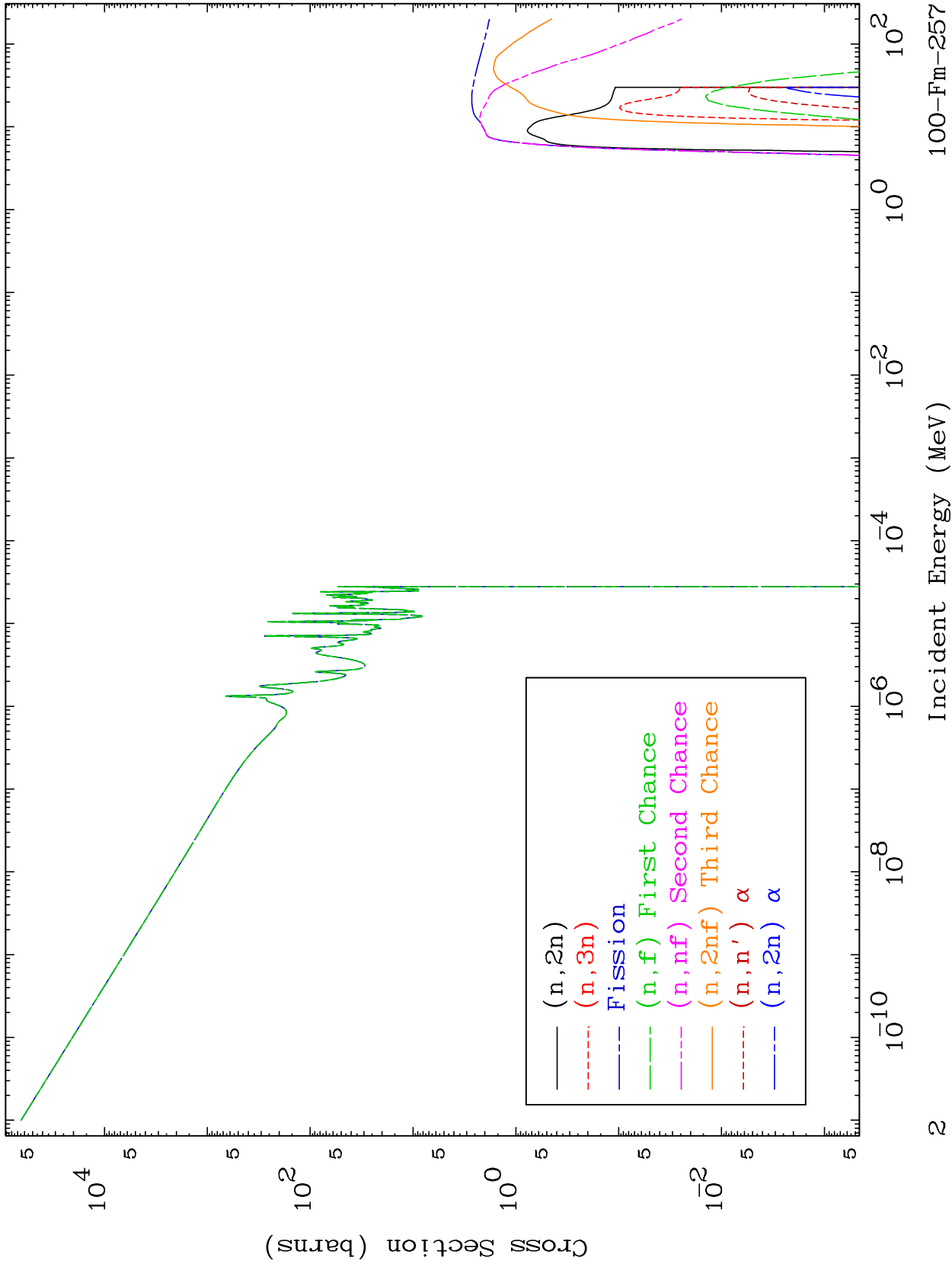
100-Fm-257



MAT 9938

Neutron Absorption  
293 Kelvin Cross Sections

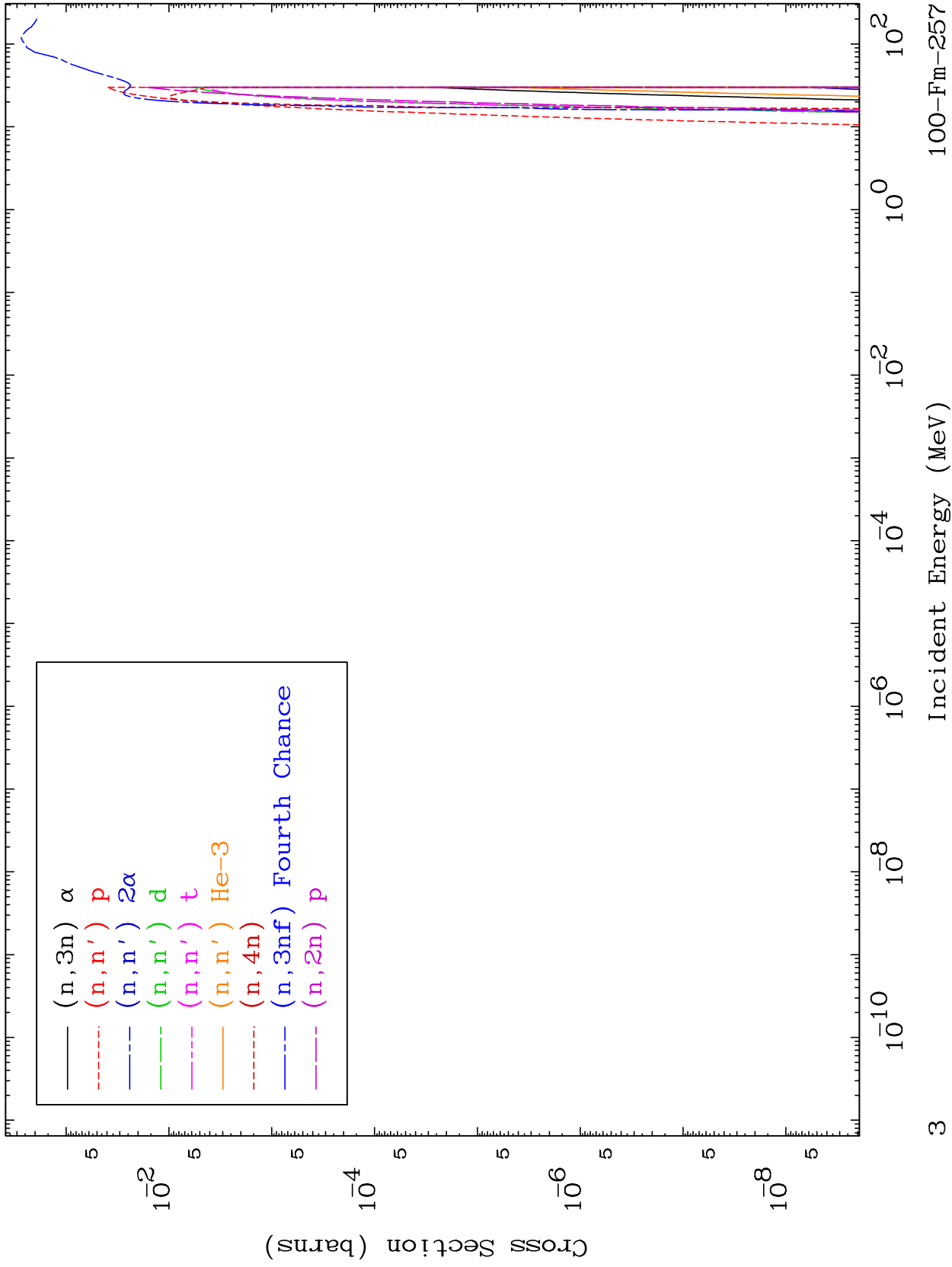
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MAT 9938

Neutron Absorption  
293 Kelvin Cross Sections

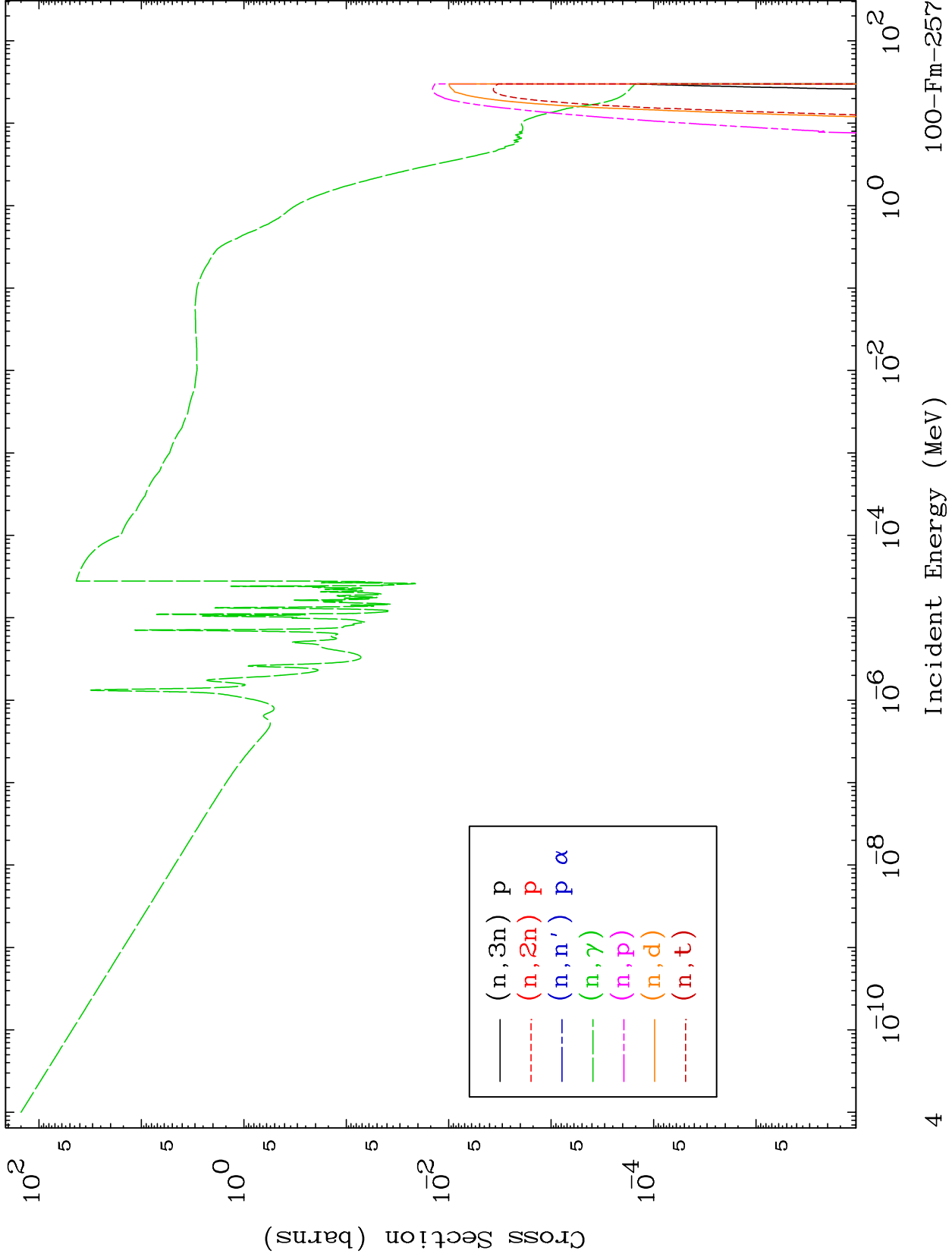
100-Fm-257



MAT 9938

Neutron Absorption  
293 Kelvin Cross Sections

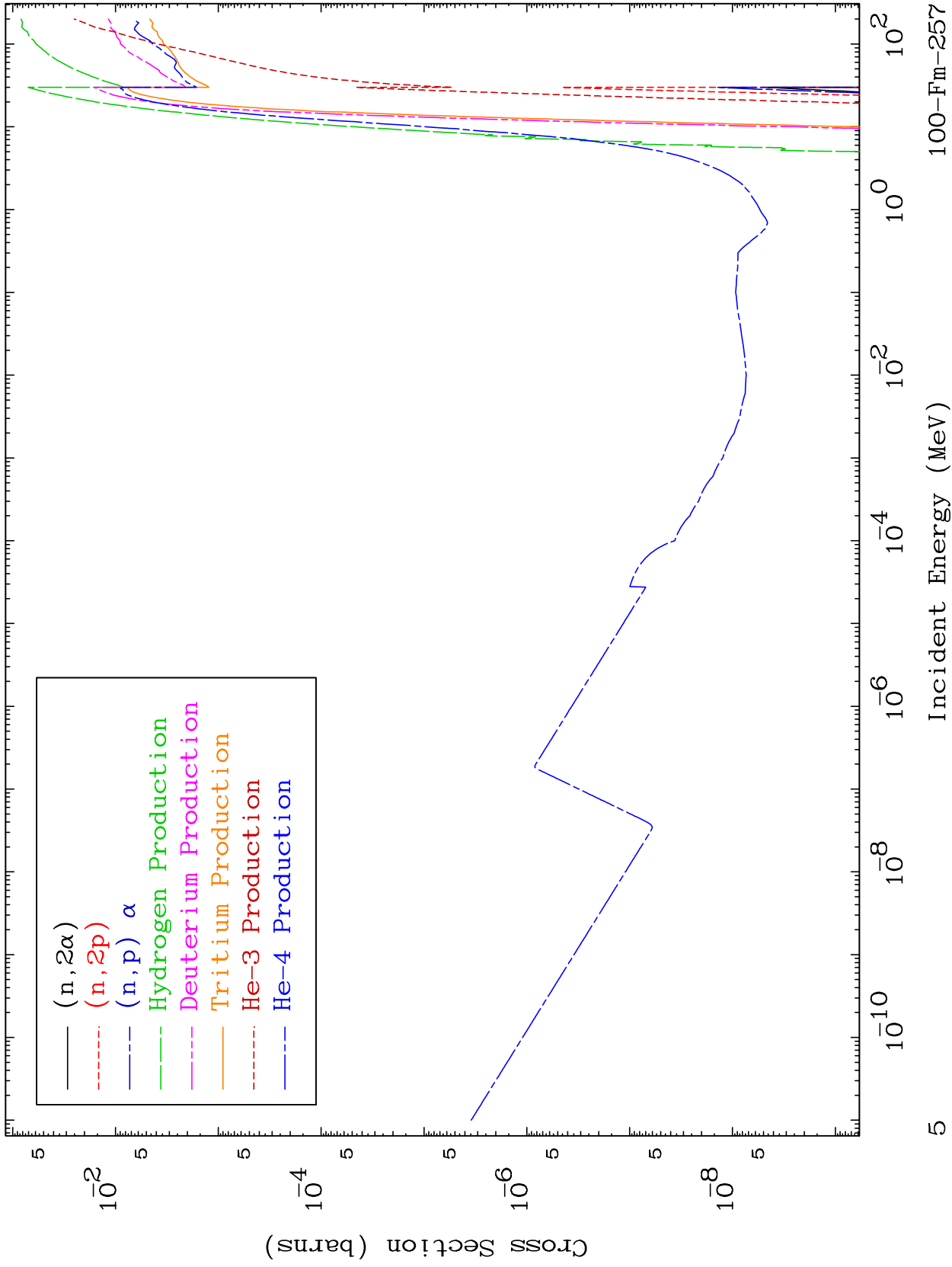
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MAT 9938

Neutron Absorption  
293 Kelvin Cross Sections

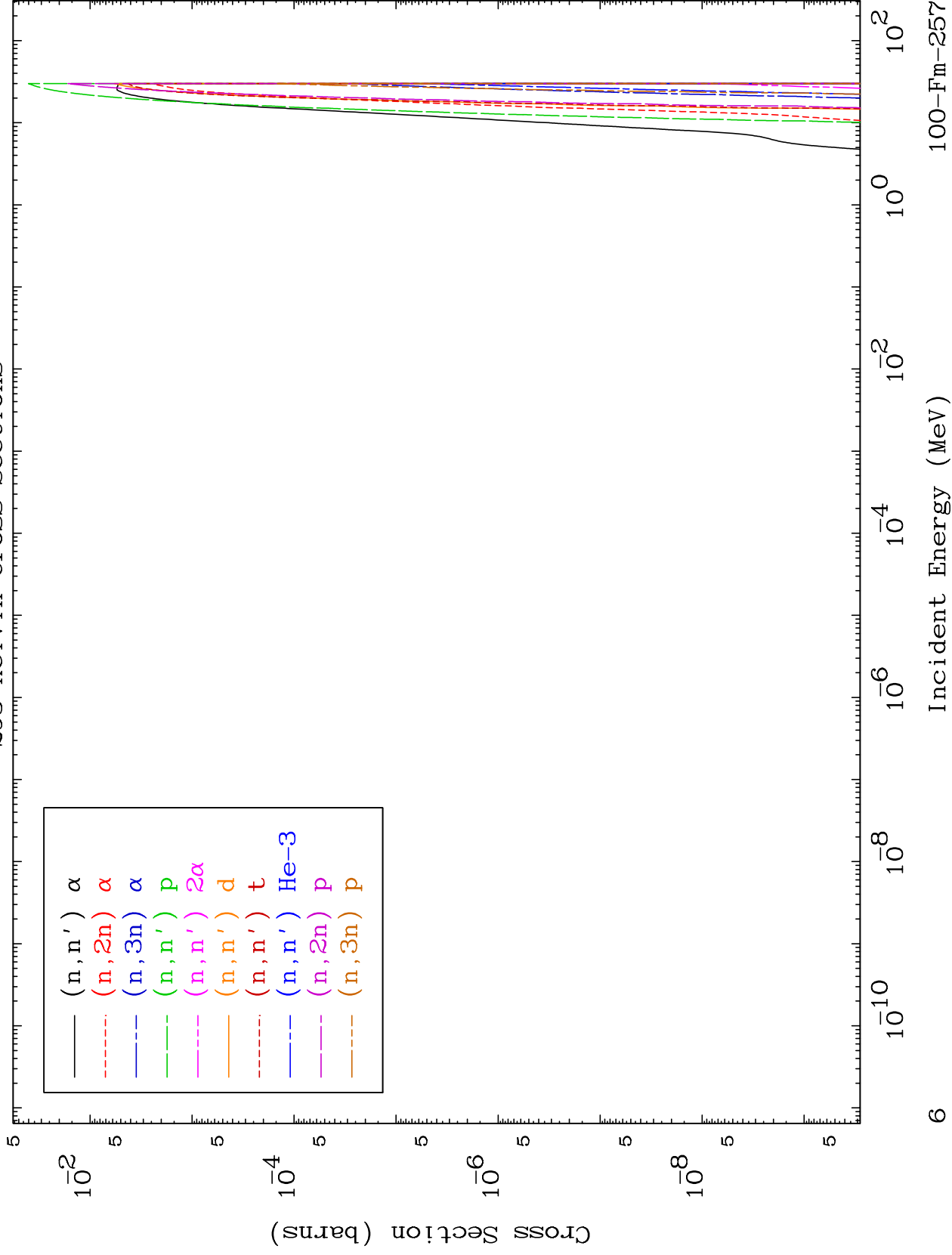
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MAT 9938

Charged Particle  
293 Kelvin Cross Sections

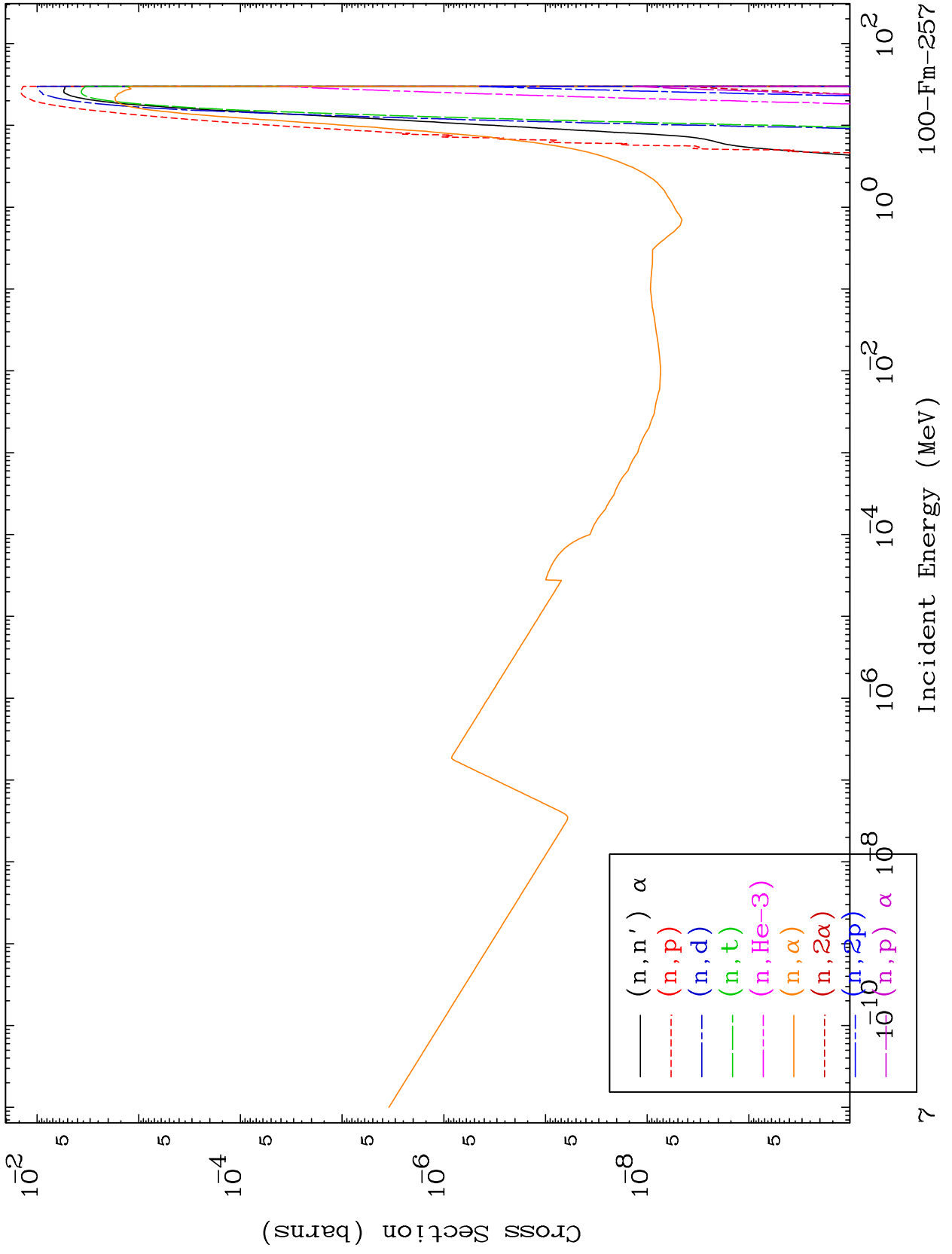
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Charged Particle  
293 Kelvin Cross Sections

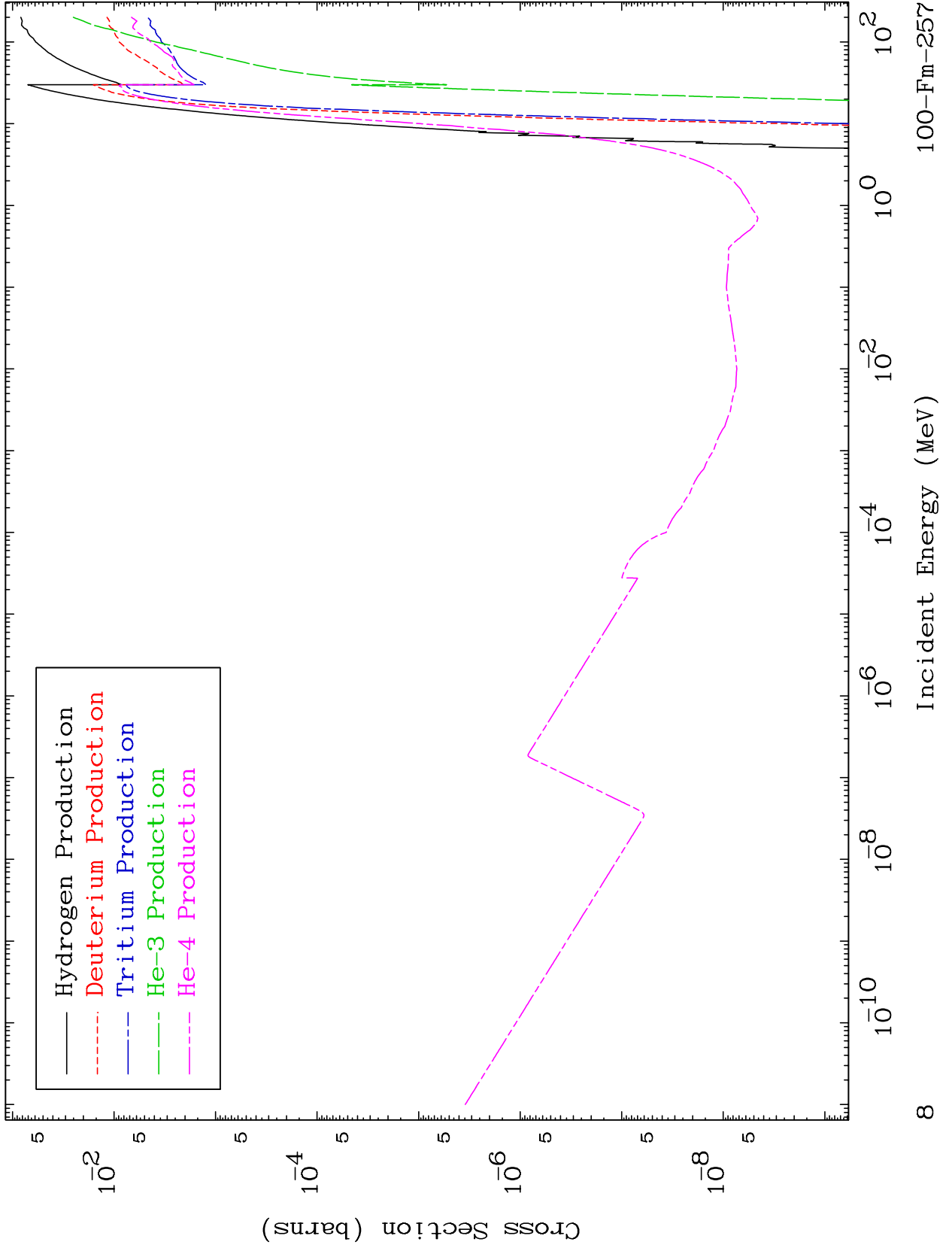
100-Fm-257



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Particle Production  
293 Kelvin Cross Sections

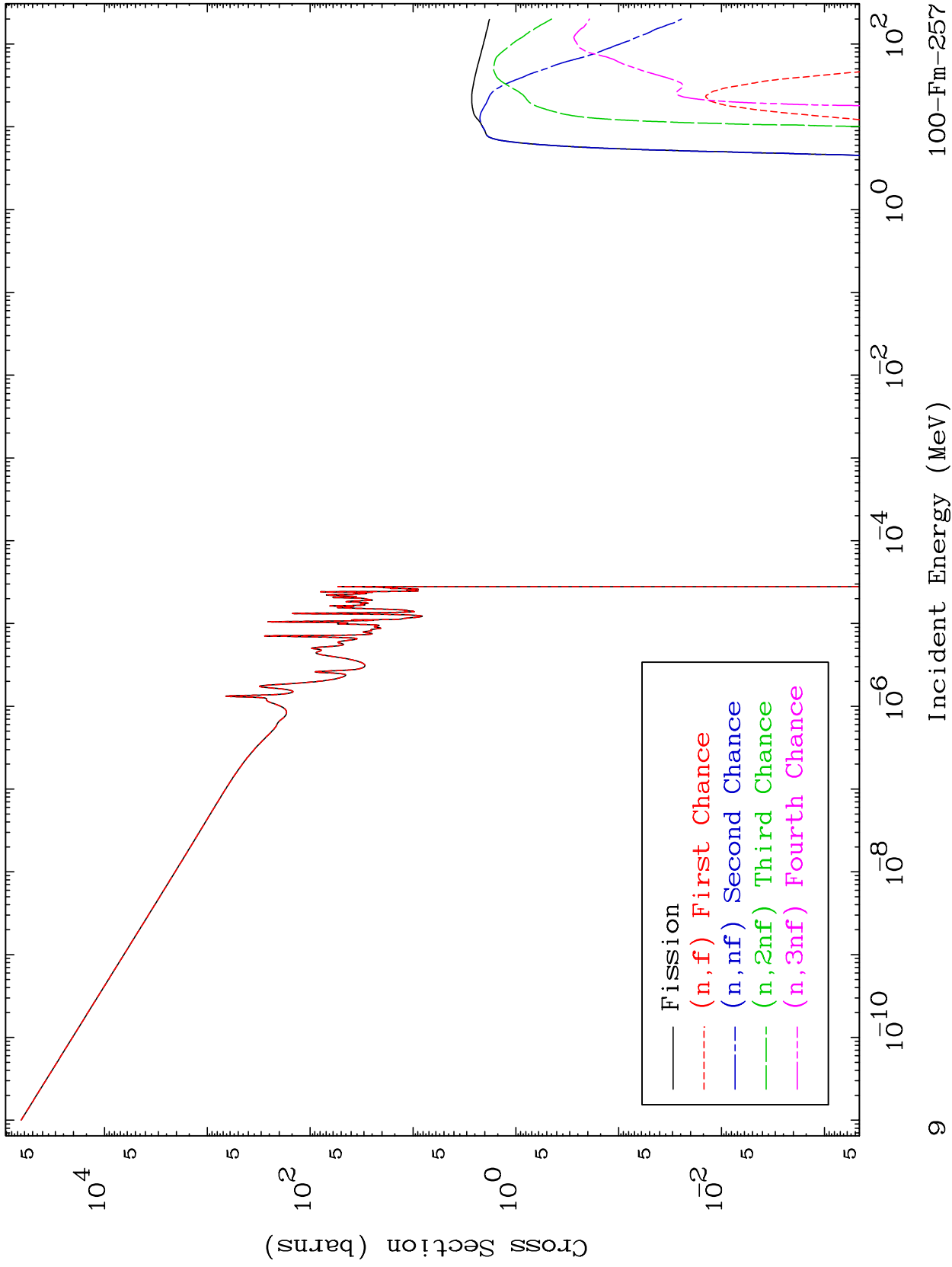
100-Fm-257



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Fission  
293 Kelvin Cross Sections

100-Fm-257

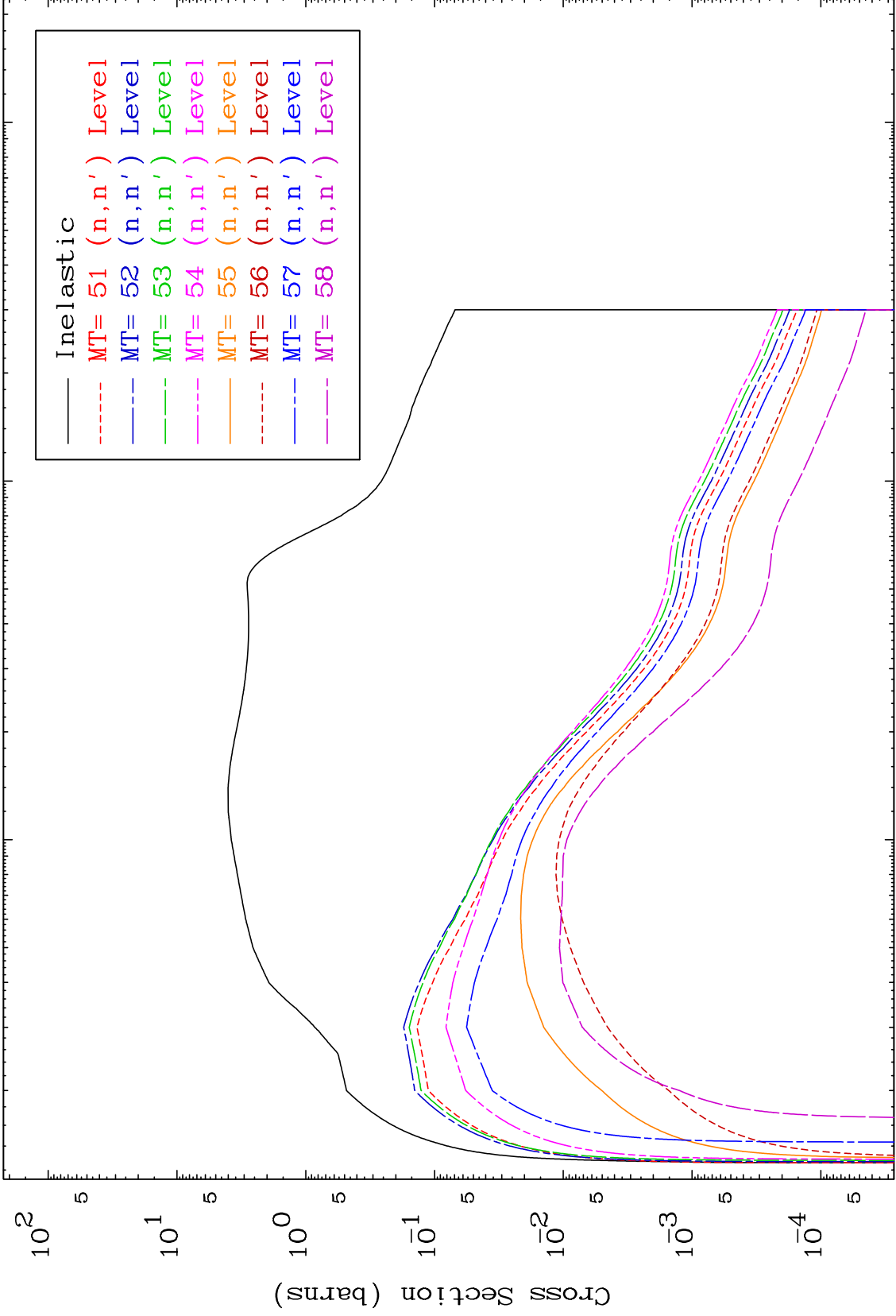


MAT 9938

(n,n') Levels

100-Fm-257

293 Kelvin Cross Sections



10

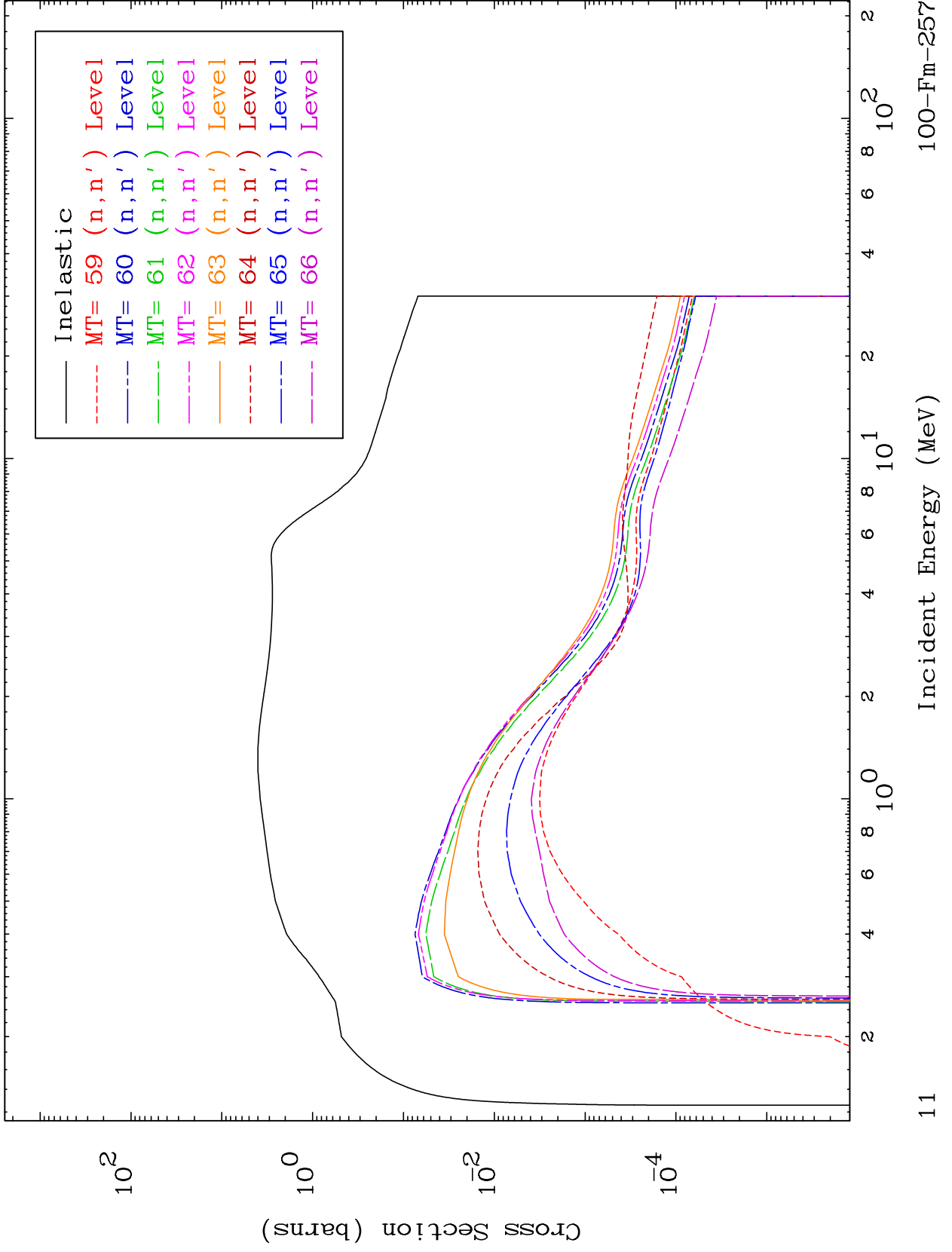
Incident Energy (MeV)

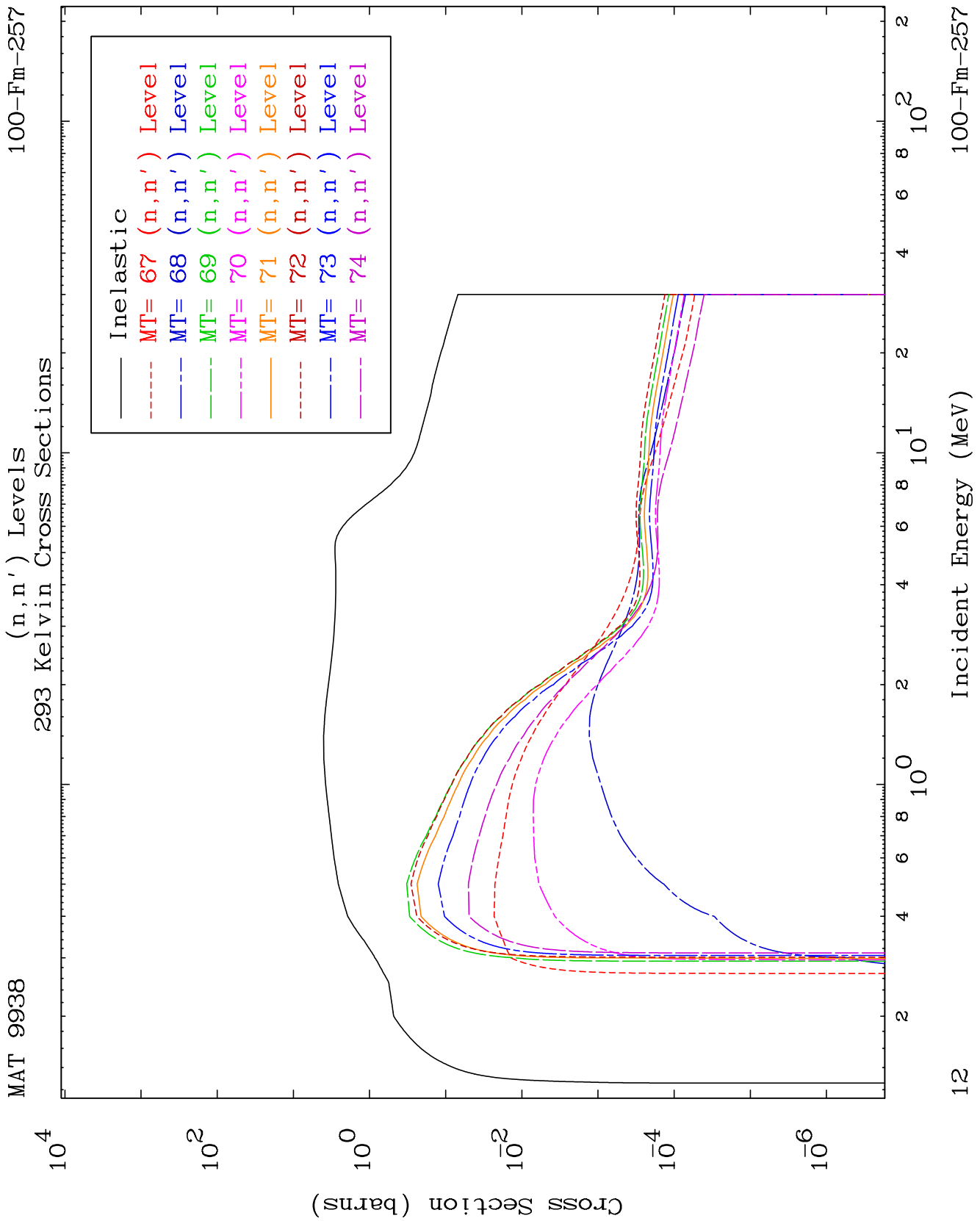
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MAT 9938

(n,n') Levels  
293 Kelvin Cross Sections

100-Fm-257

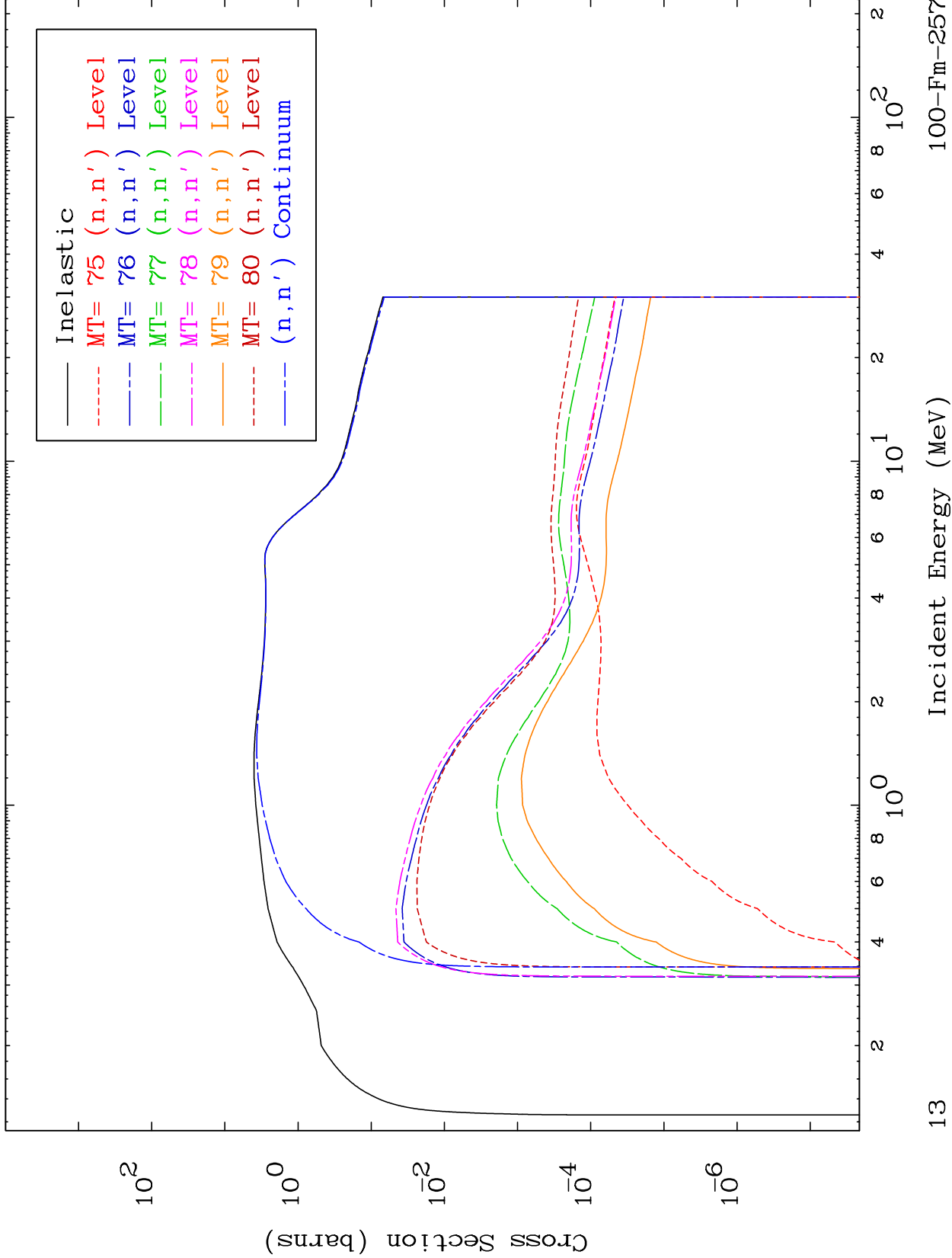




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(n,n') Levels  
293 Kelvin Cross Sections

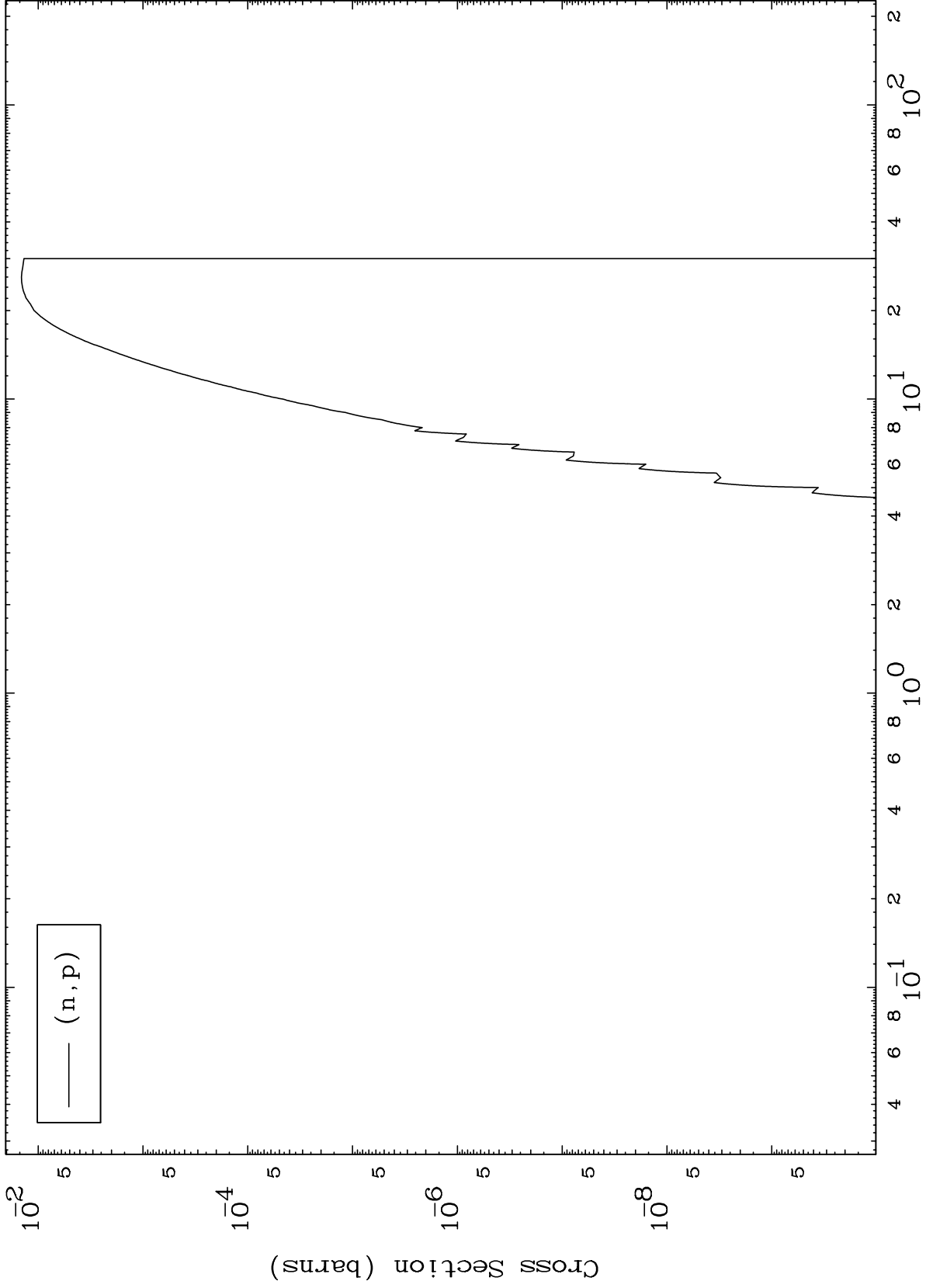
100-Fm-257



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(n,p) Levels  
293 Kelvin Cross Sections

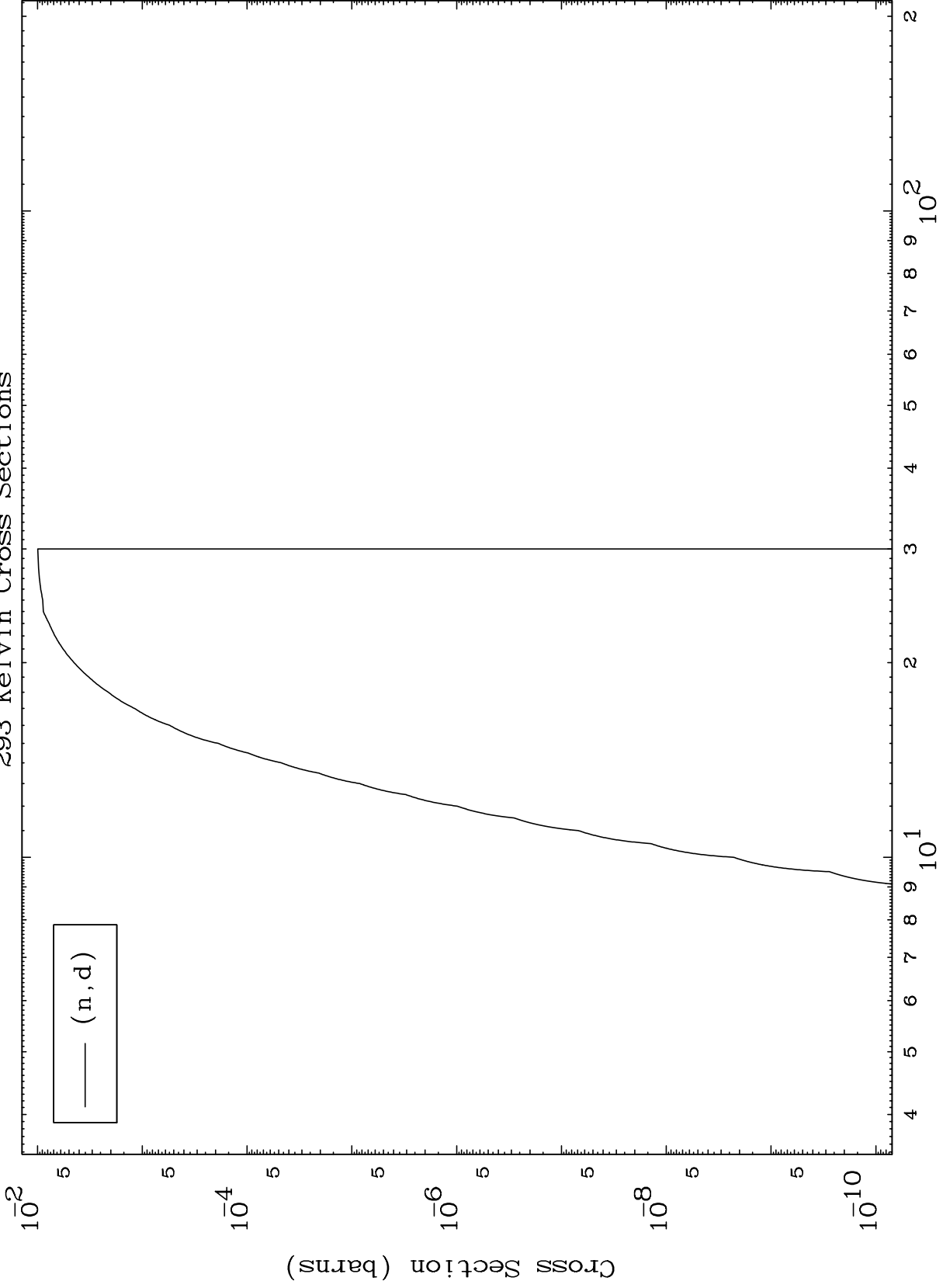
100-Fm-257



MAT 9938

(n,d) Levels  
293 Kelvin Cross Sections

100-Fm-257



15

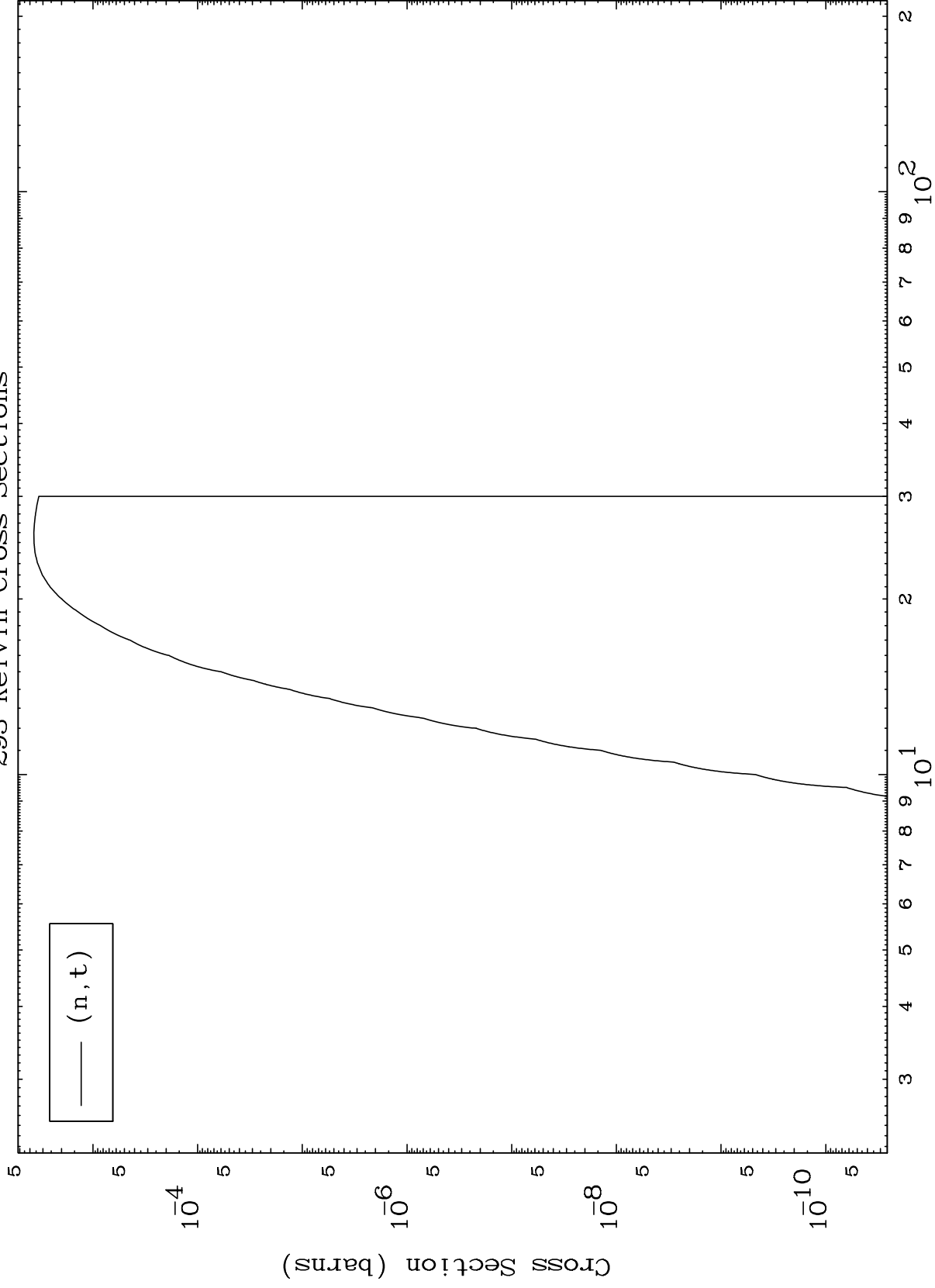
Incident Energy (MeV)

100-Fm-257

MAT 9938

(n,t) Levels  
293 Kelvin Cross Sections

100-Fm-257



16

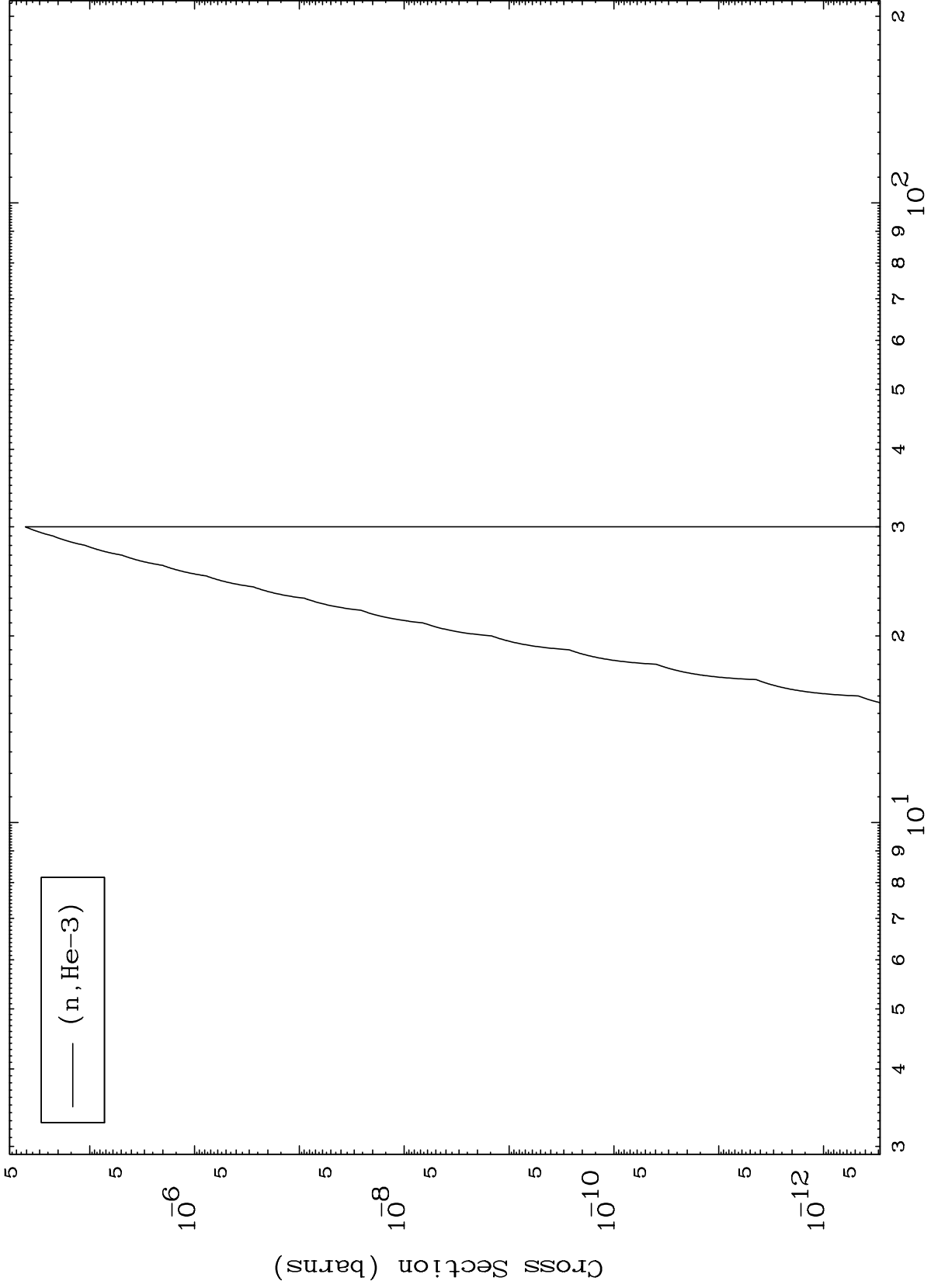
Incident Energy (MeV)

100-Fm-257

MAT 9938

(n,He3) Levels  
293 Kelvin Cross Sections

100-Fm-257



17

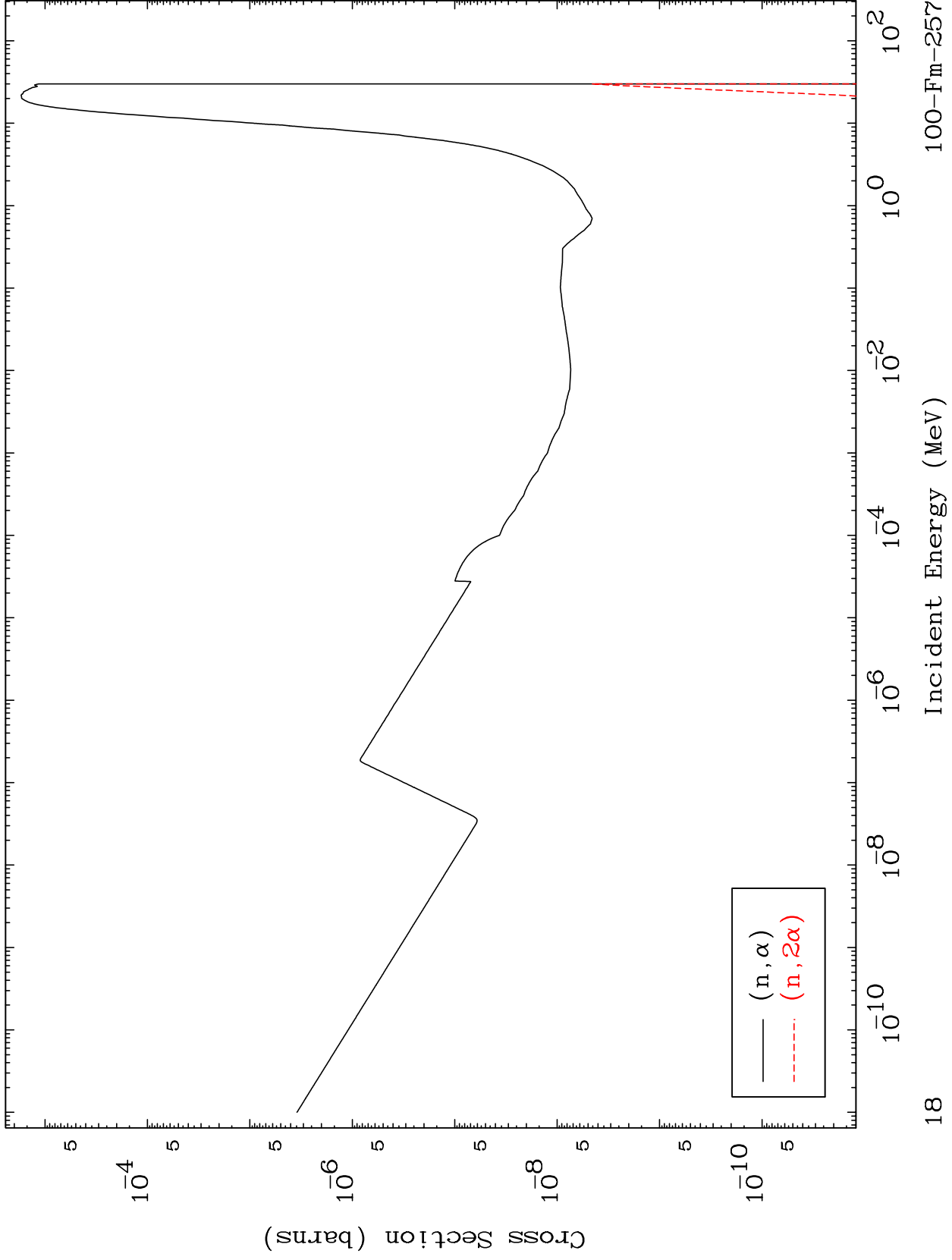
Incident Energy (MeV)

100-Fm-257

MAT 9938

(n,α) Levels  
293 Kelvin Cross Sections

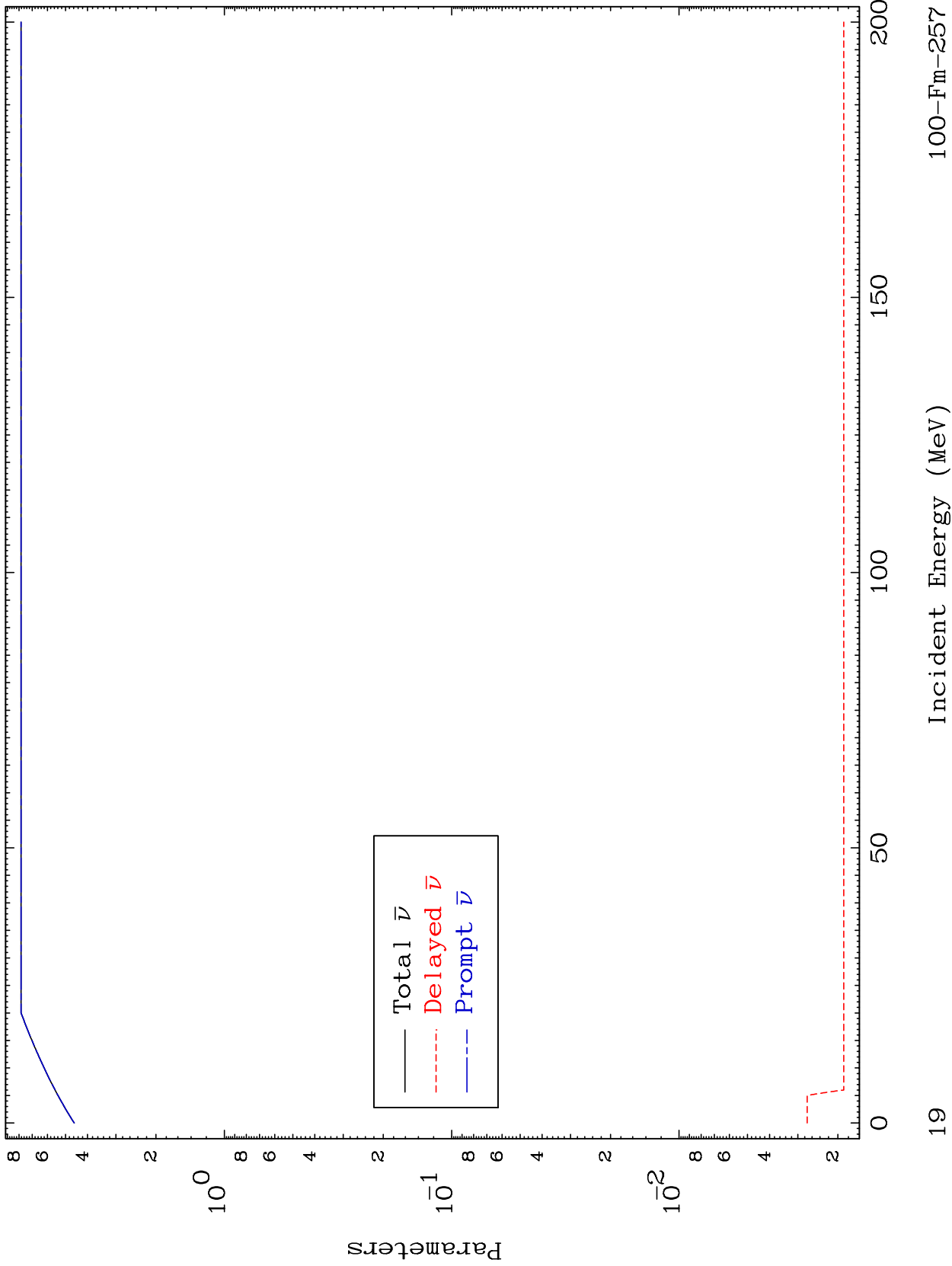
100-Fm-257



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Energy Release  
Parameters

100-Fm-257



19

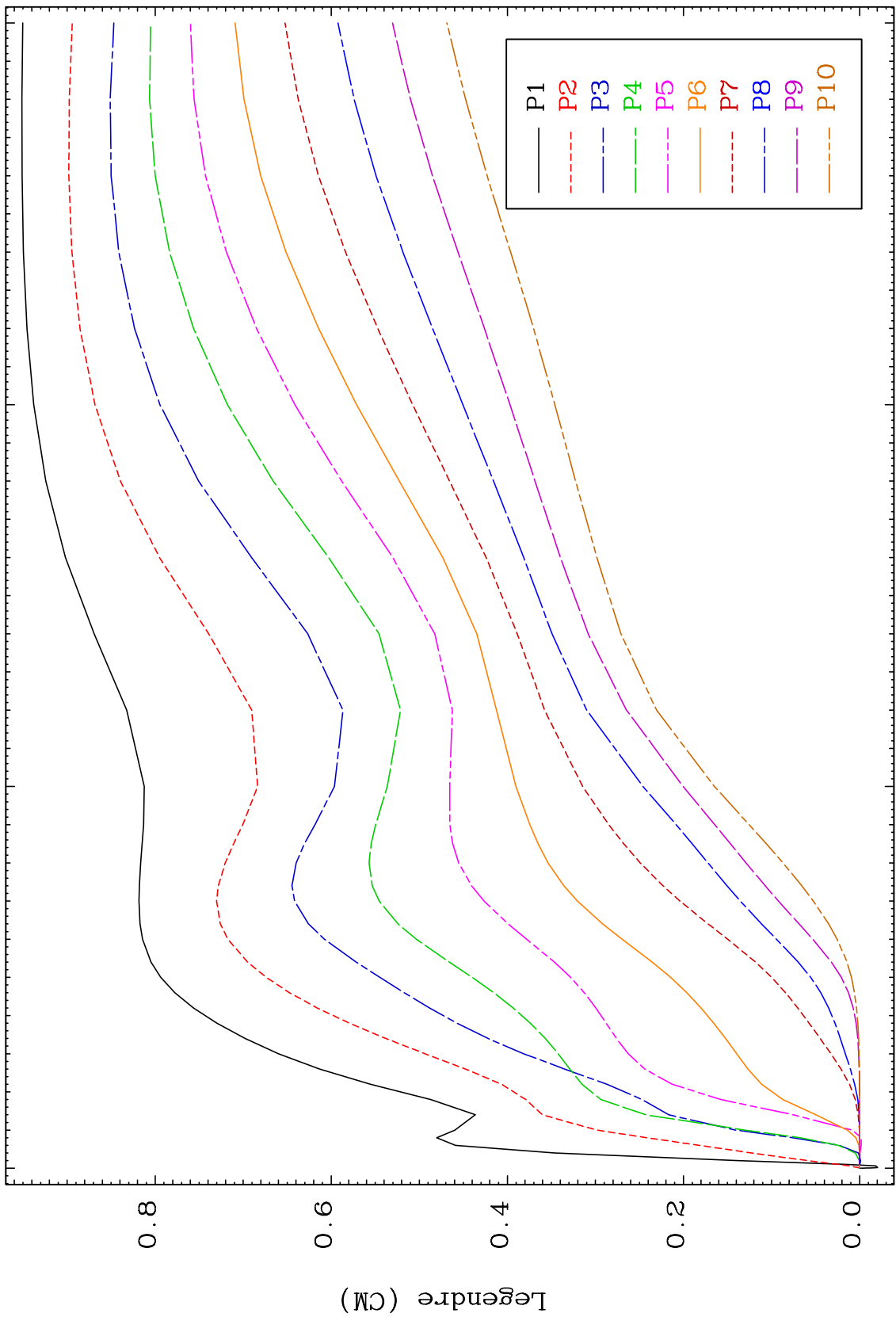
Incident Energy (MeV)

100-Fm-257

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Elastic Legendre Coefficients

100-Fm-257



20

Incident Energy (MeV)

100-Fm-257

0

10

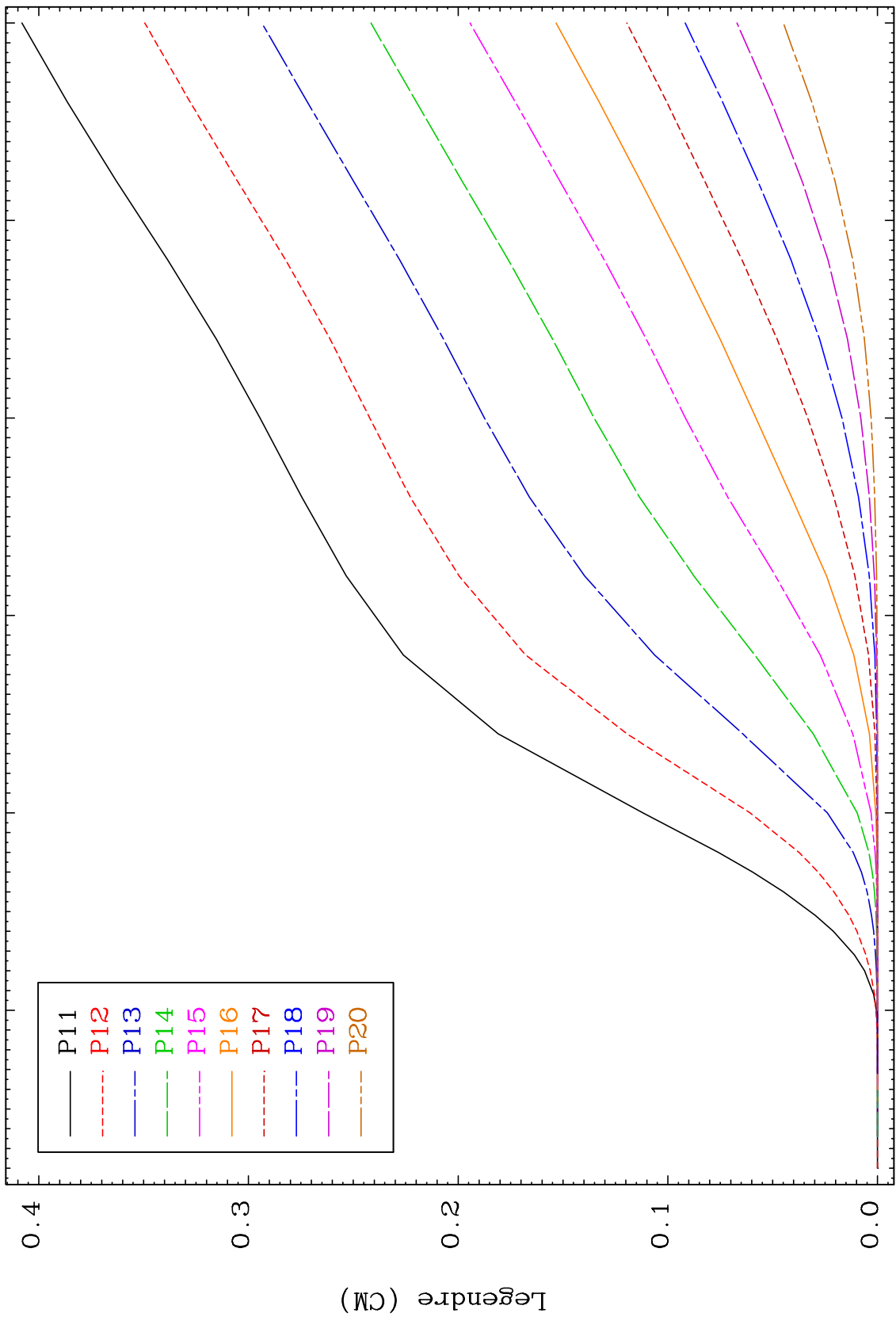
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30

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Elastic Legendre Coefficients

100-Fm-257



21

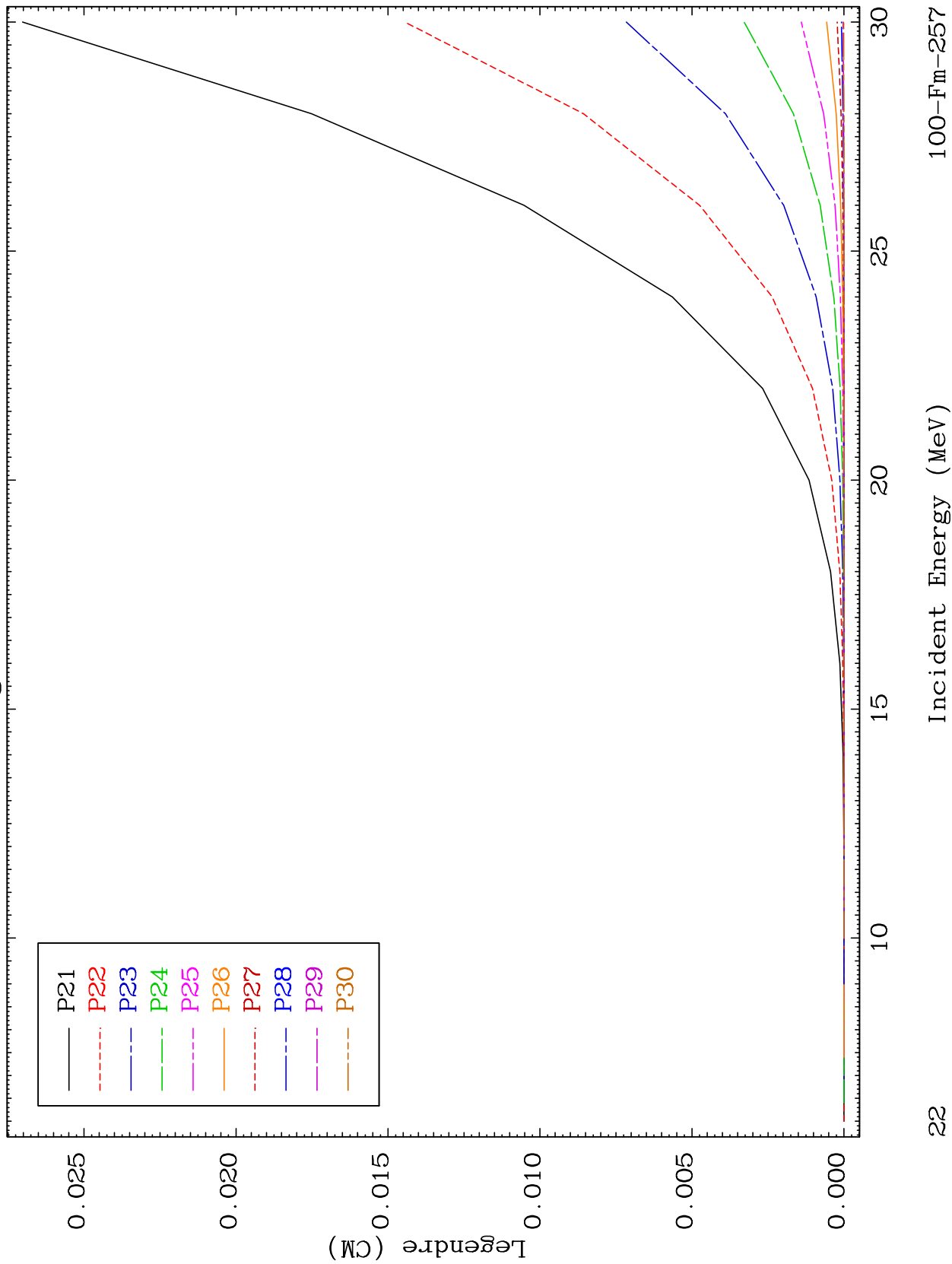
Incident Energy (MeV)

100-Fm-257

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### Elastic Legendre Coefficients

100-Fm-257



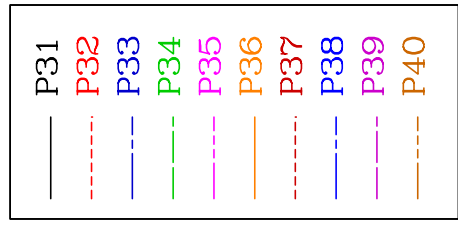
22

100-Fm-257

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Elastic Legendre Coefficients

100-Fm-257



$\times 10^{-6}$

3

Legendre (CM)

1

0

15

20

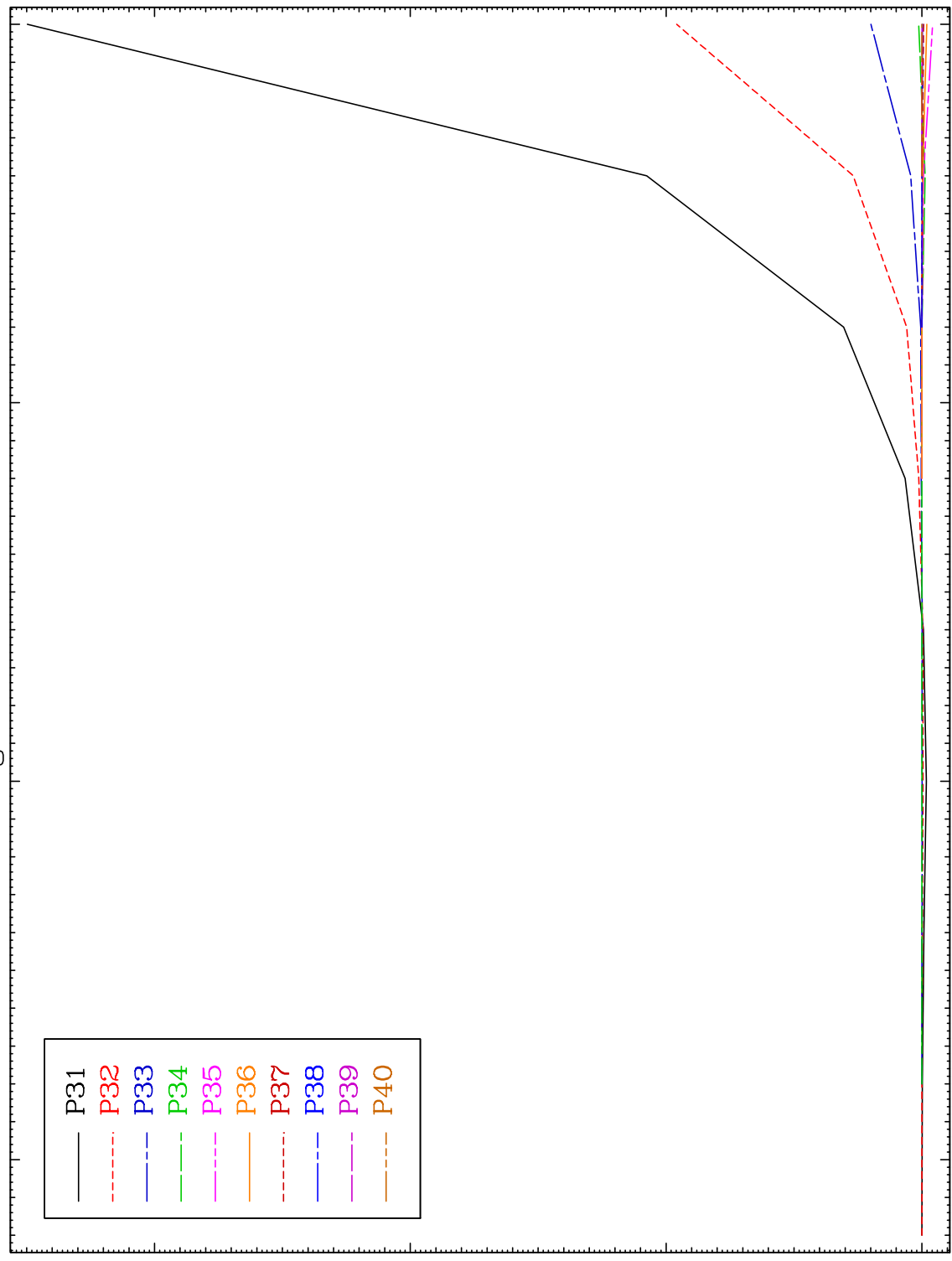
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30

23

Incident Energy (MeV)

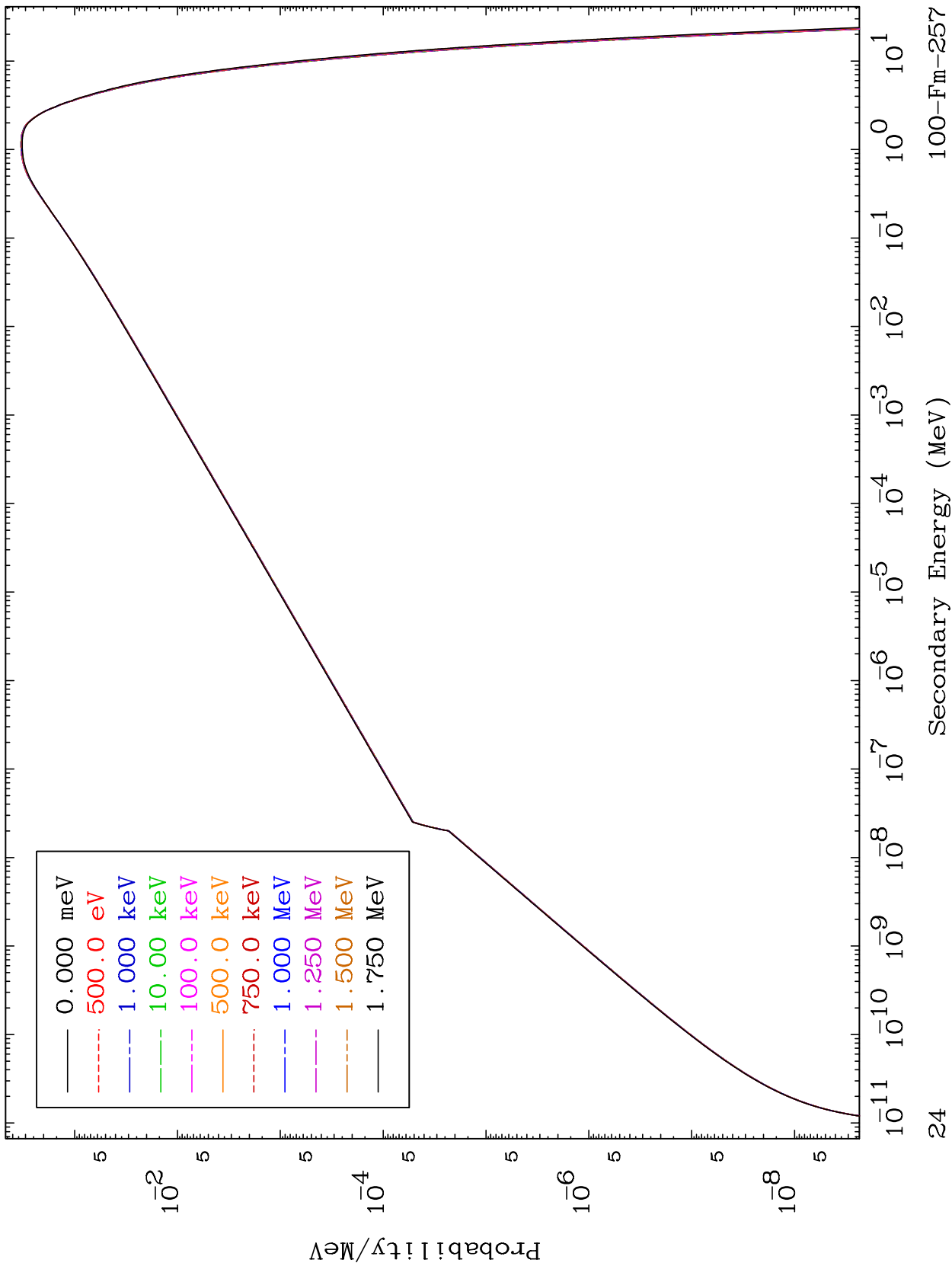
100-Fm-257



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### Fission Energy Distributions

100-Fm-257



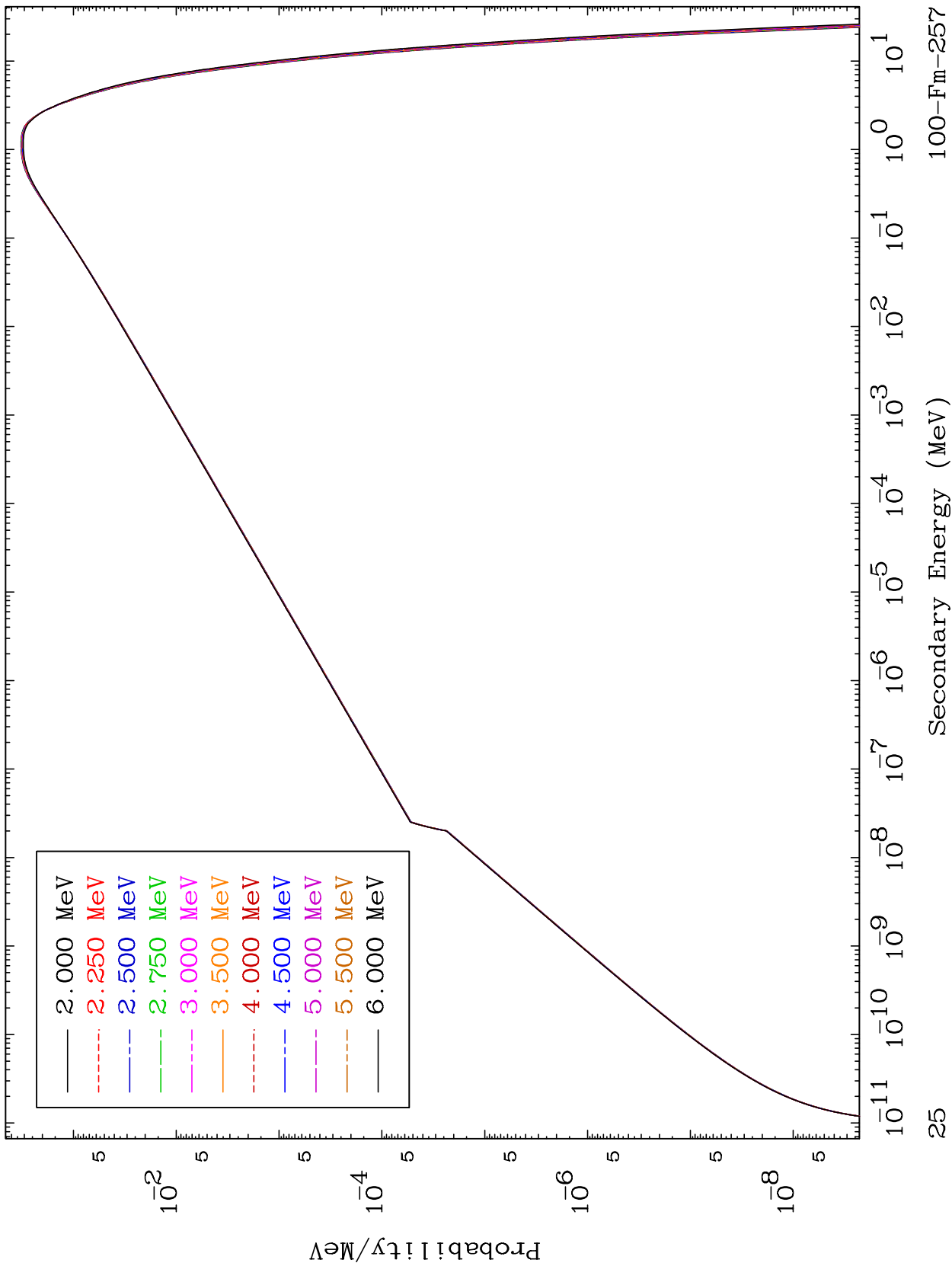
100-Fm-257

Secondary Energy (MeV)

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### Fission Energy Distributions

100-Fm-257



100-Fm-257

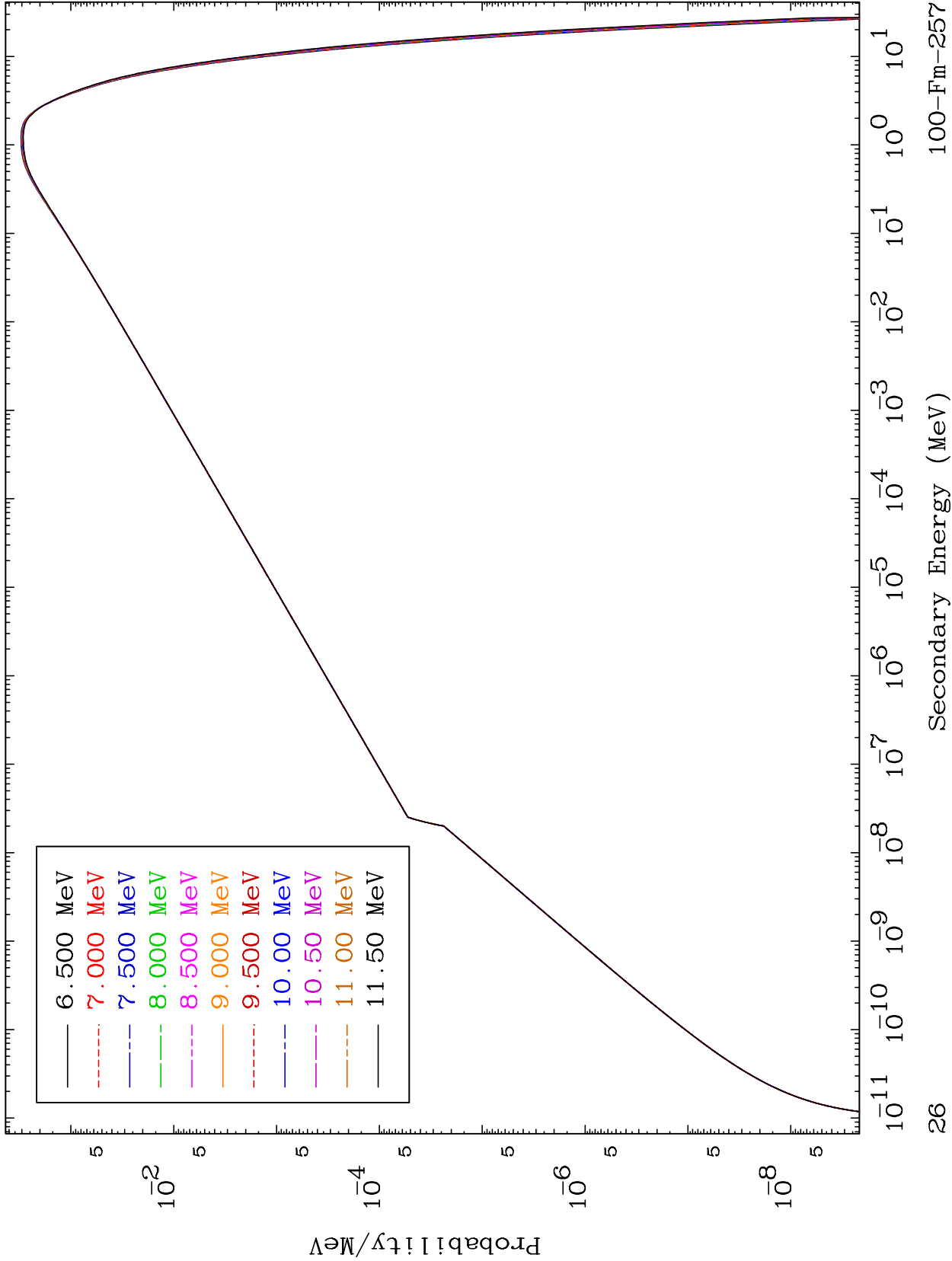
Secondary Energy (MeV)

25

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### Fission Energy Distributions

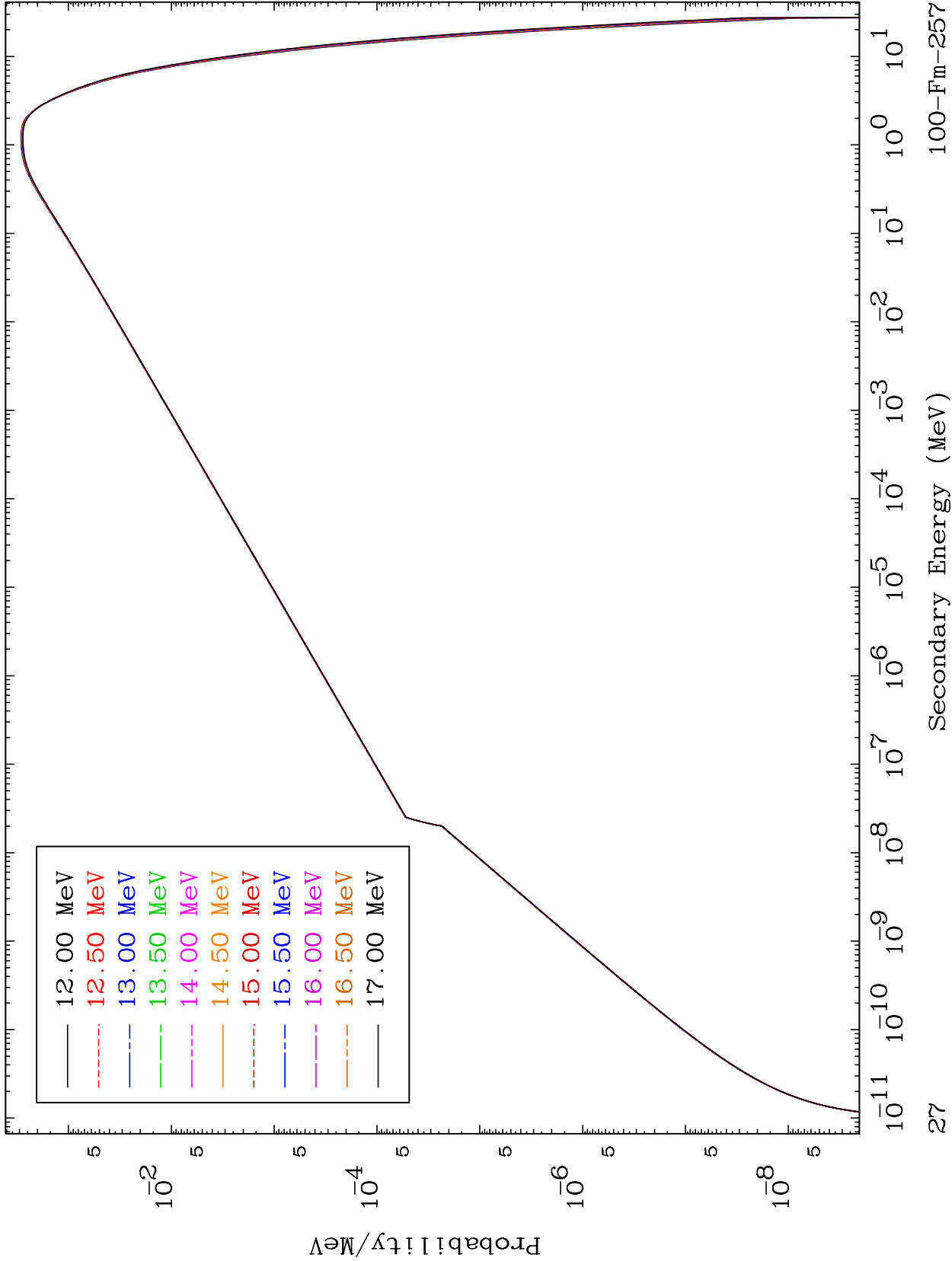
100-Fm-257



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### Fission Energy Distributions

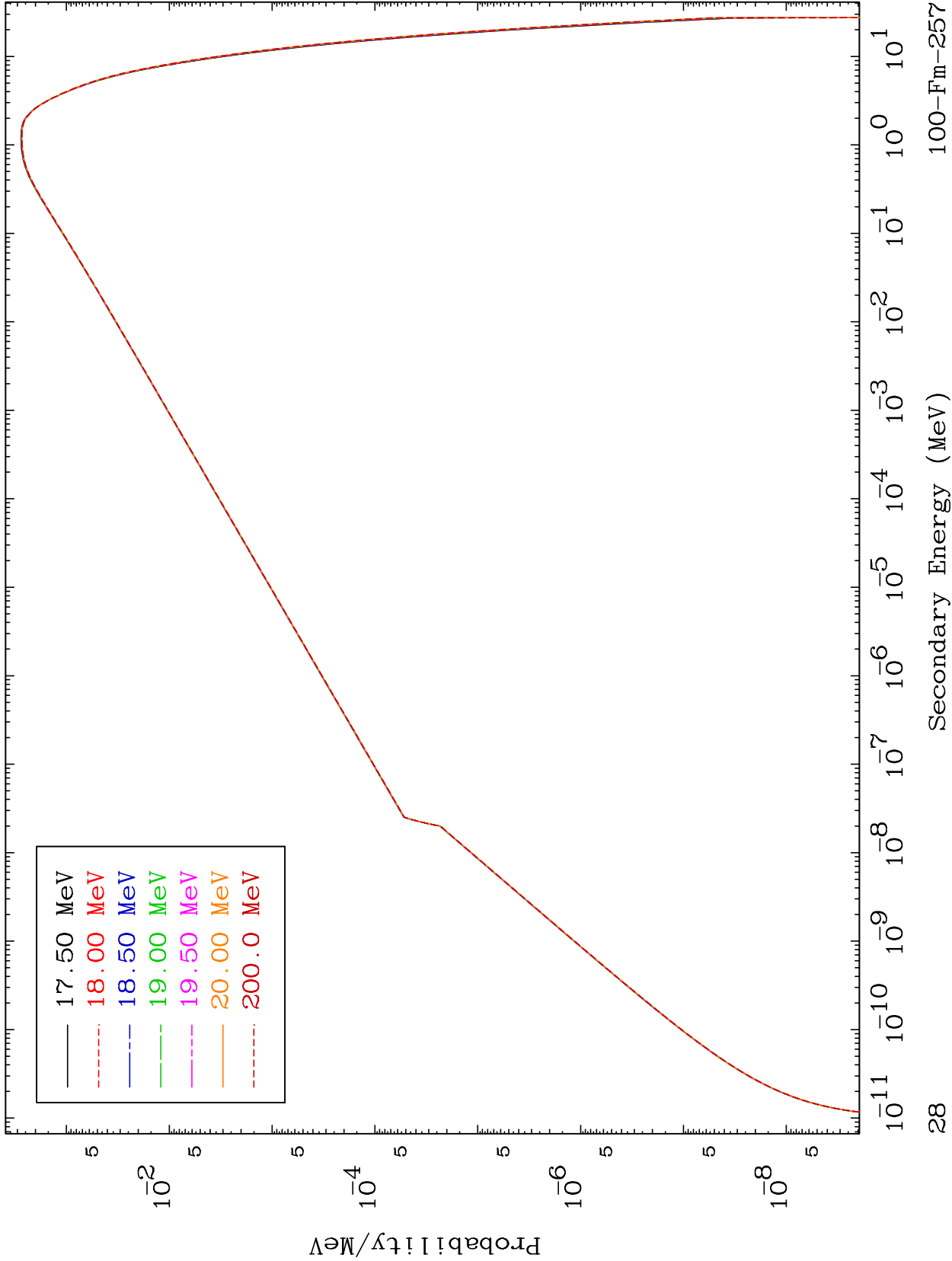
100-Fm-257



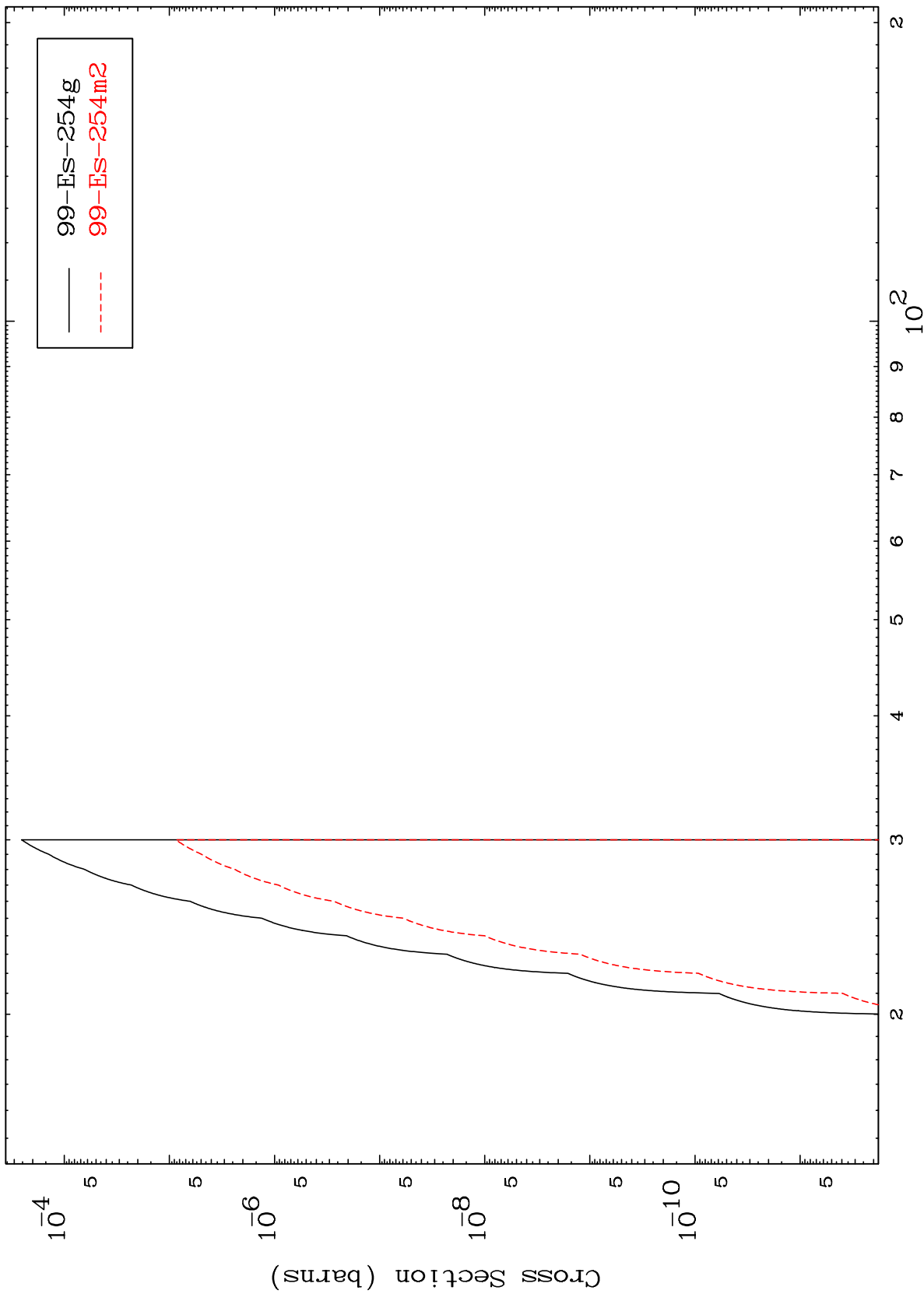
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### Fission Energy Distributions

100-Fm-257



Radionuclide Production Cross Section



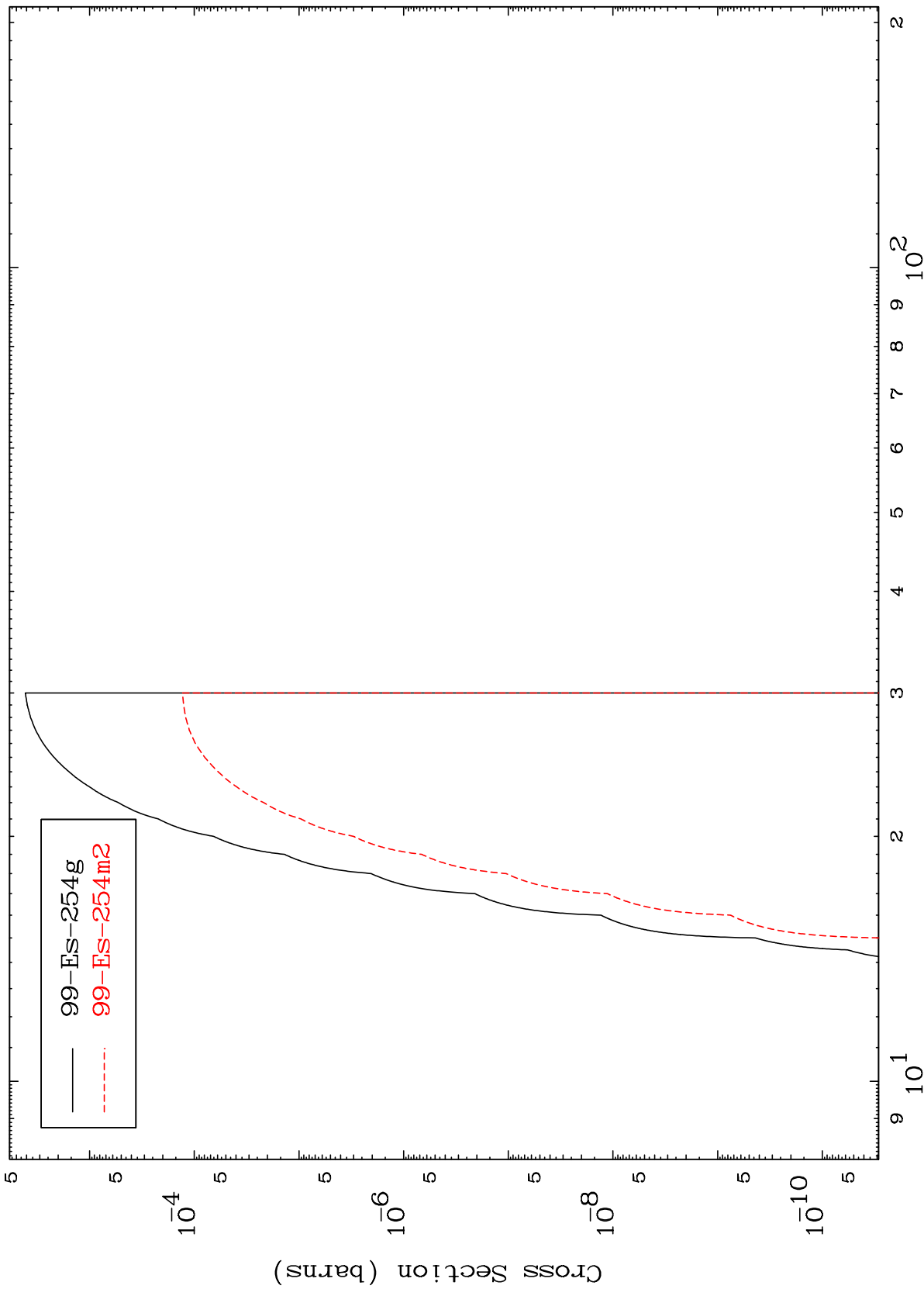
99-Es-254g  
99-Es-254m2

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(n,n') t

100-Fm-257

Radionuclide Production Cross Section



30

Incident Energy (MeV)

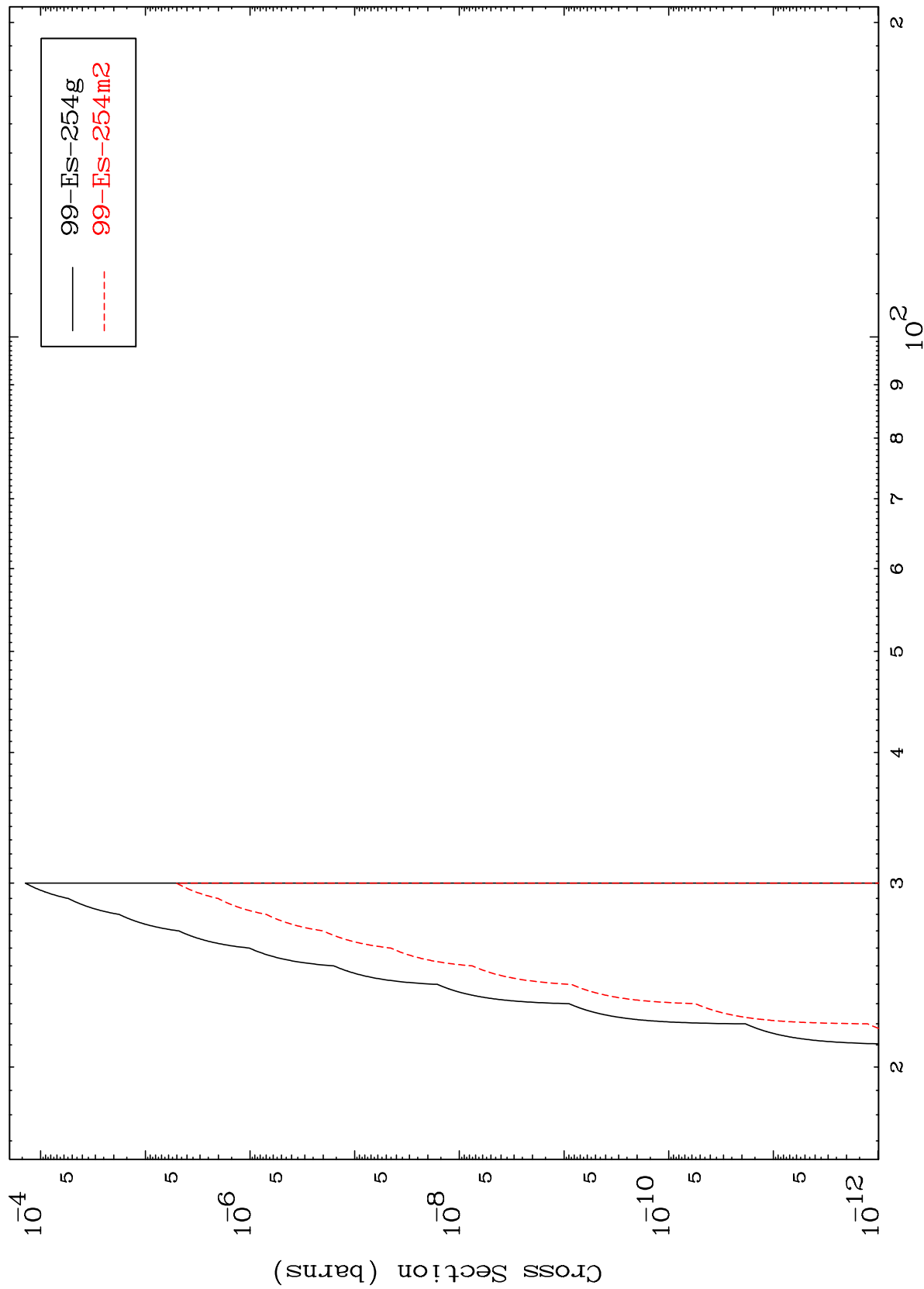
100-Fm-257

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(n,3n) p

100-Fm-257

Radionuclide Production Cross Section



99-Es-254g  
99-Es-254m2

Incident Energy (MeV)

100-Fm-257

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