

Program EVALPLOT
(Version 2021-1)

by

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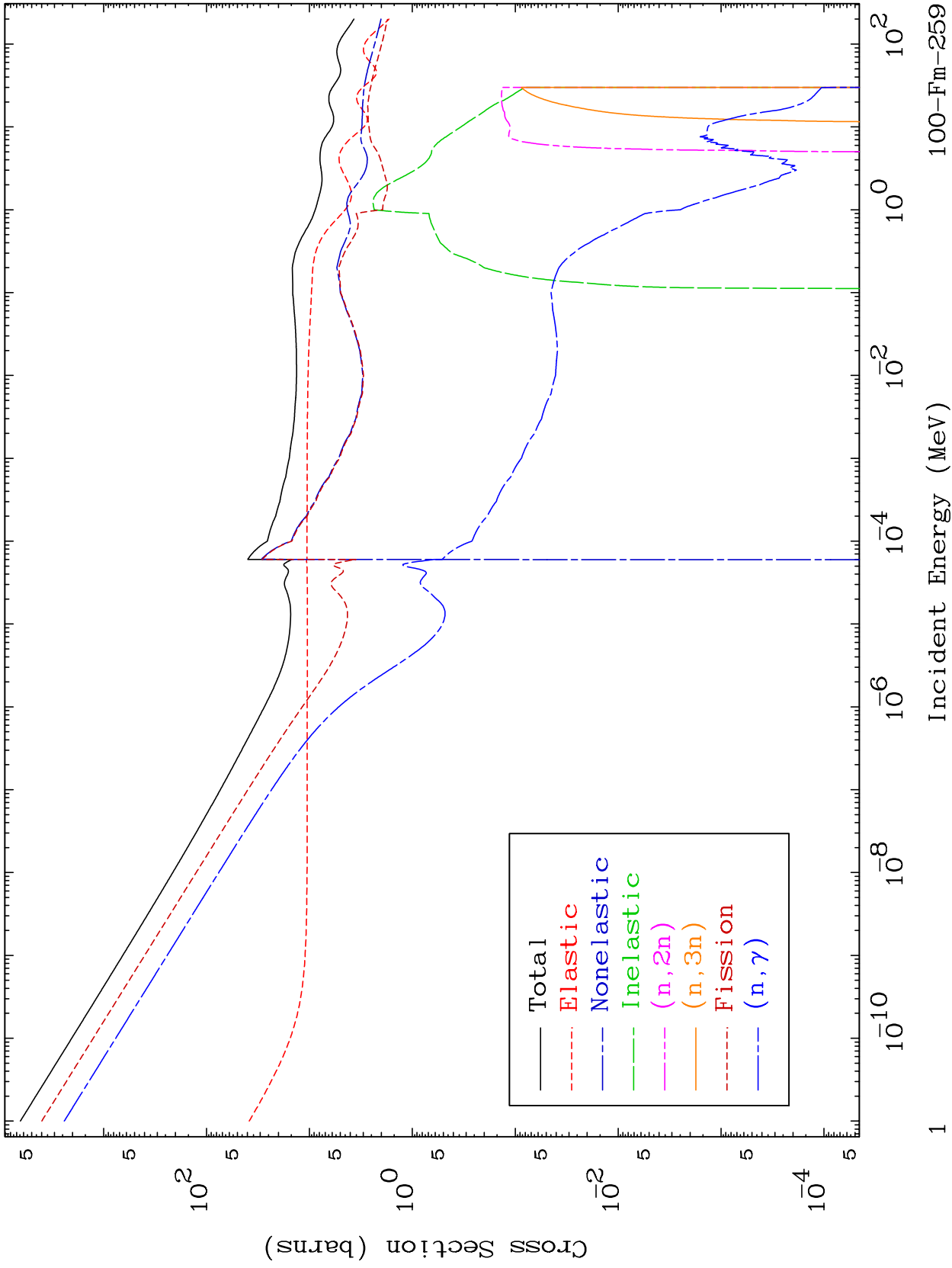
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9940

Neutron Major
293 Kelvin Cross Sections

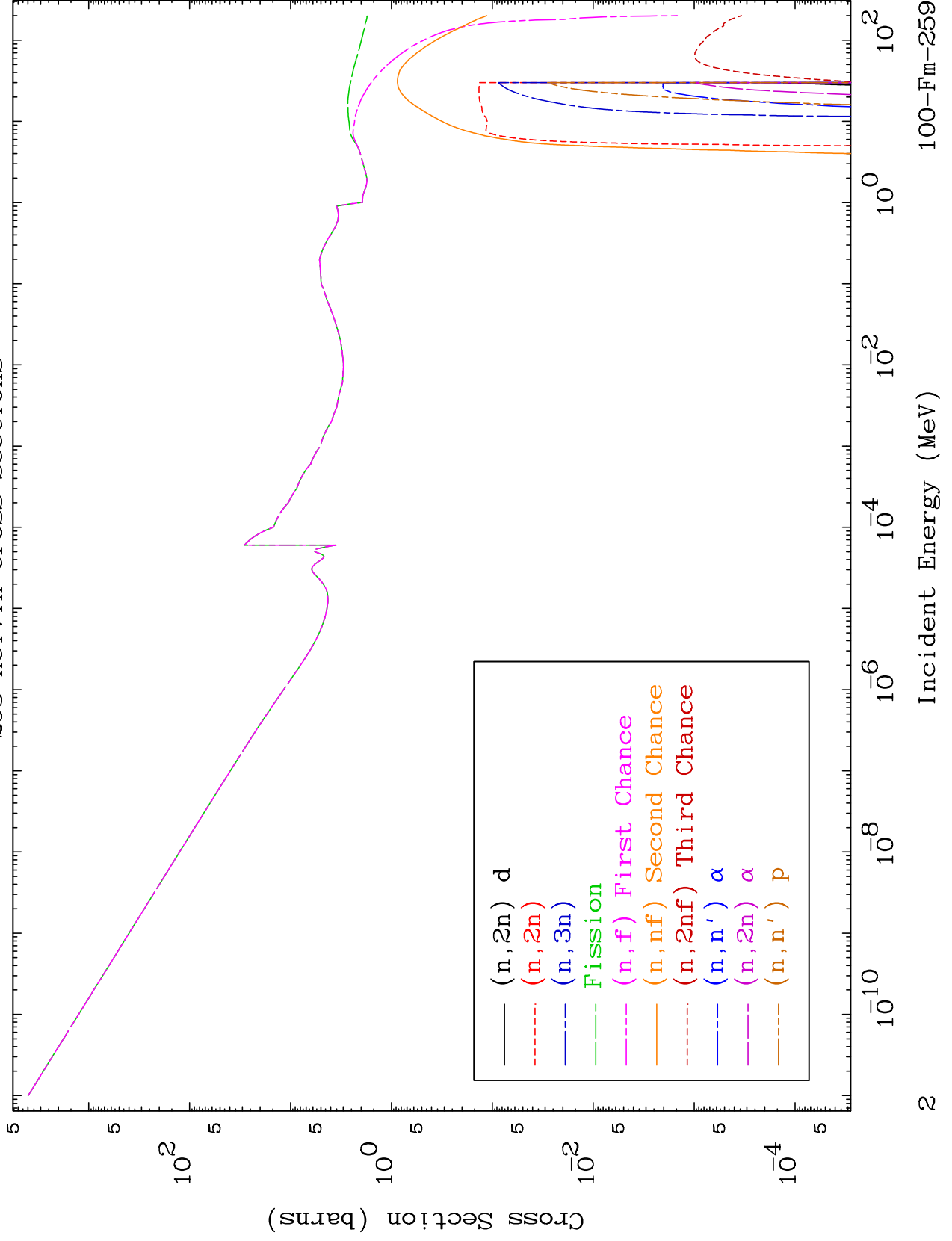
100-Fm-259



MAT 9940

Neutron Absorption
293 Kelvin Cross Sections

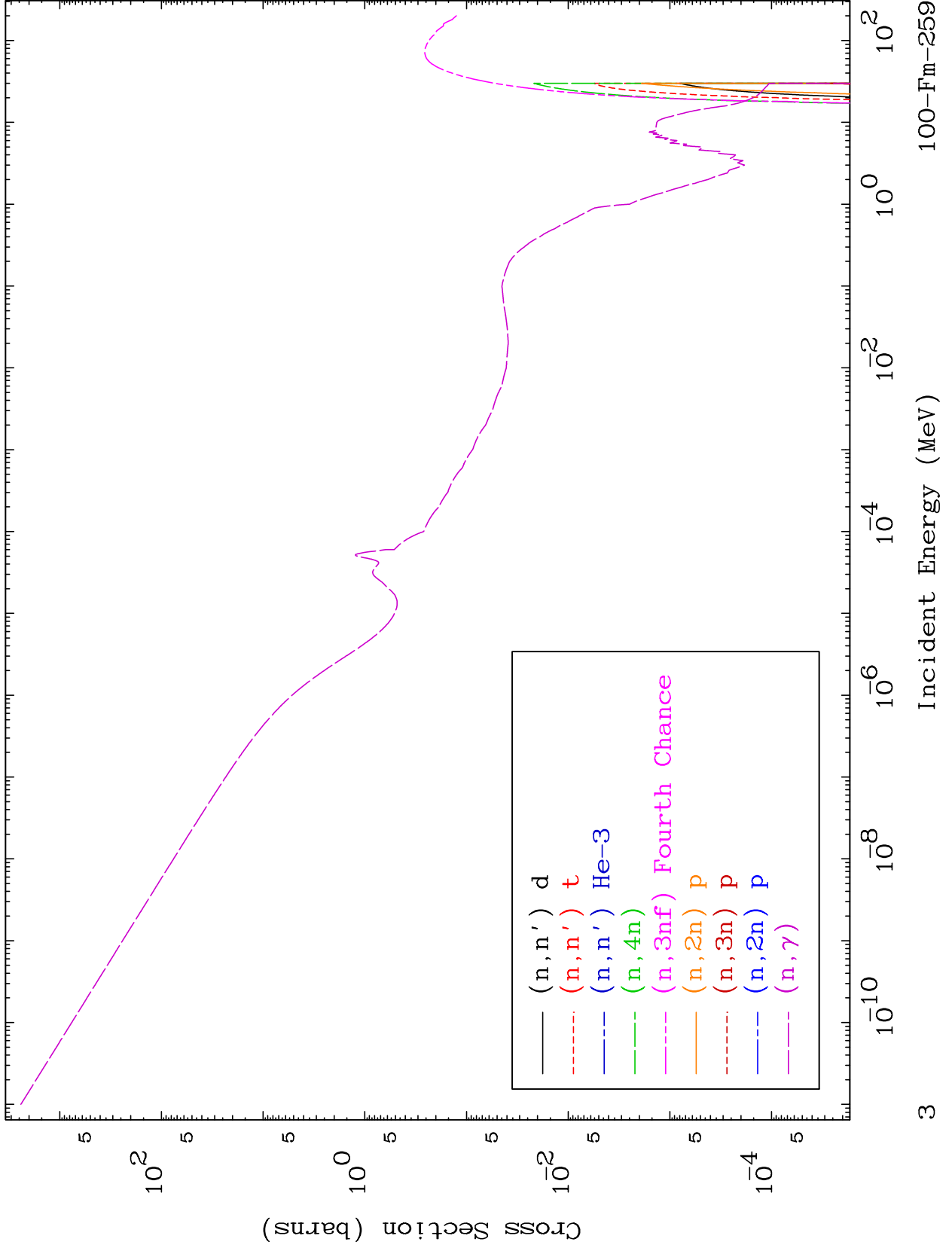
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Neutron Absorption
293 Kelvin Cross Sections

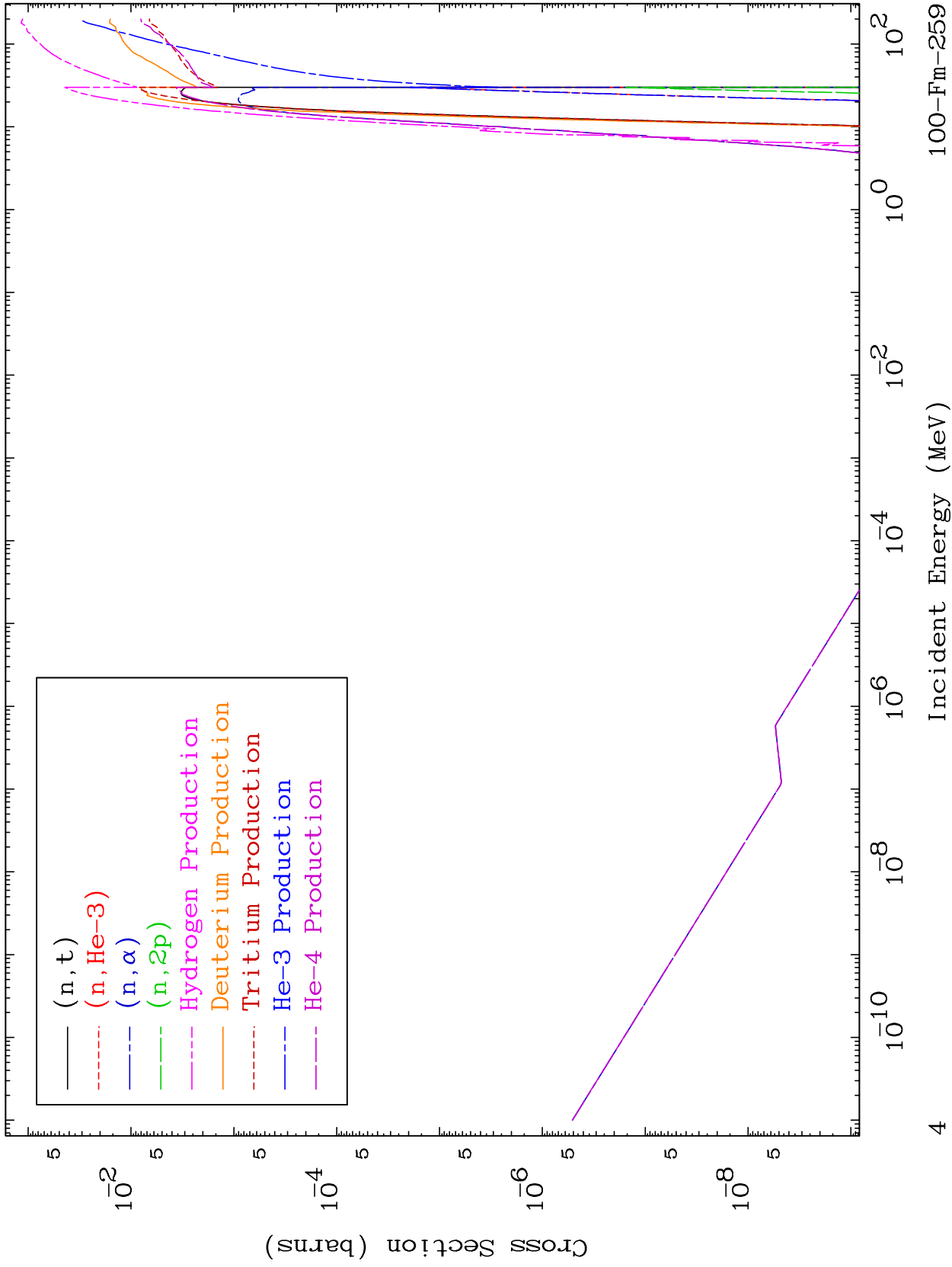
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Neutron Absorption
293 Kelvin Cross Sections

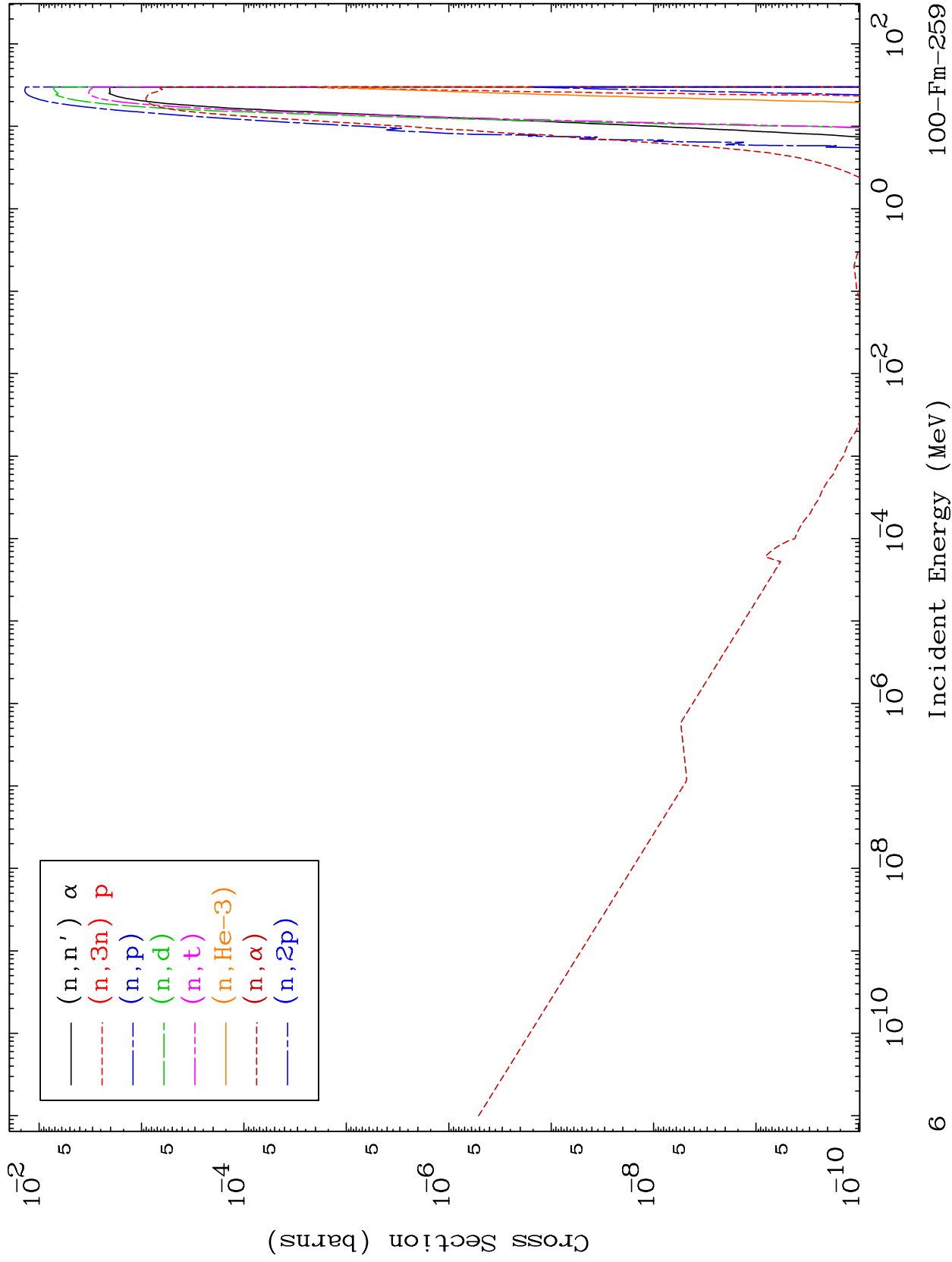
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Charged Particle
293 Kelvin Cross Sections

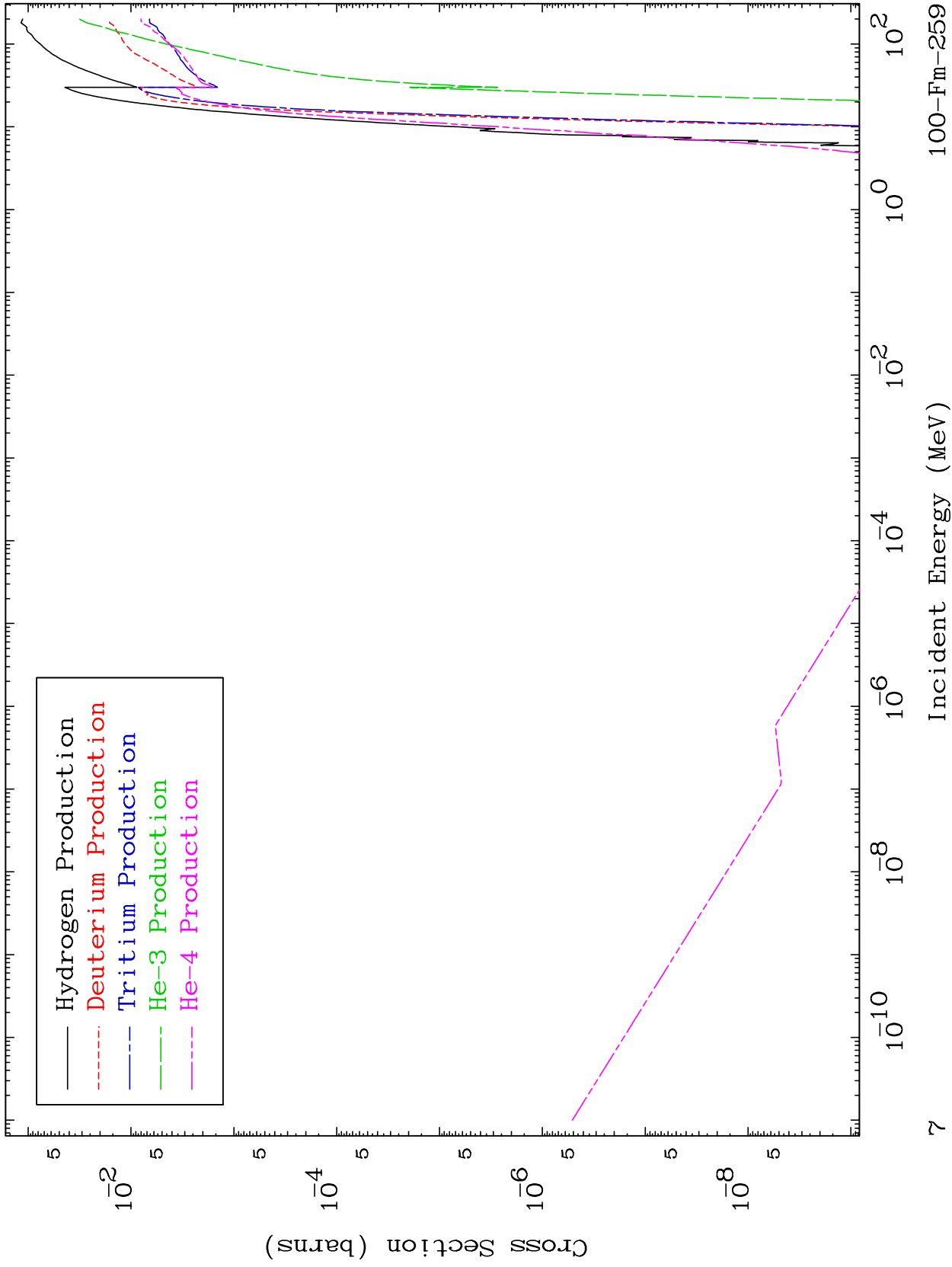
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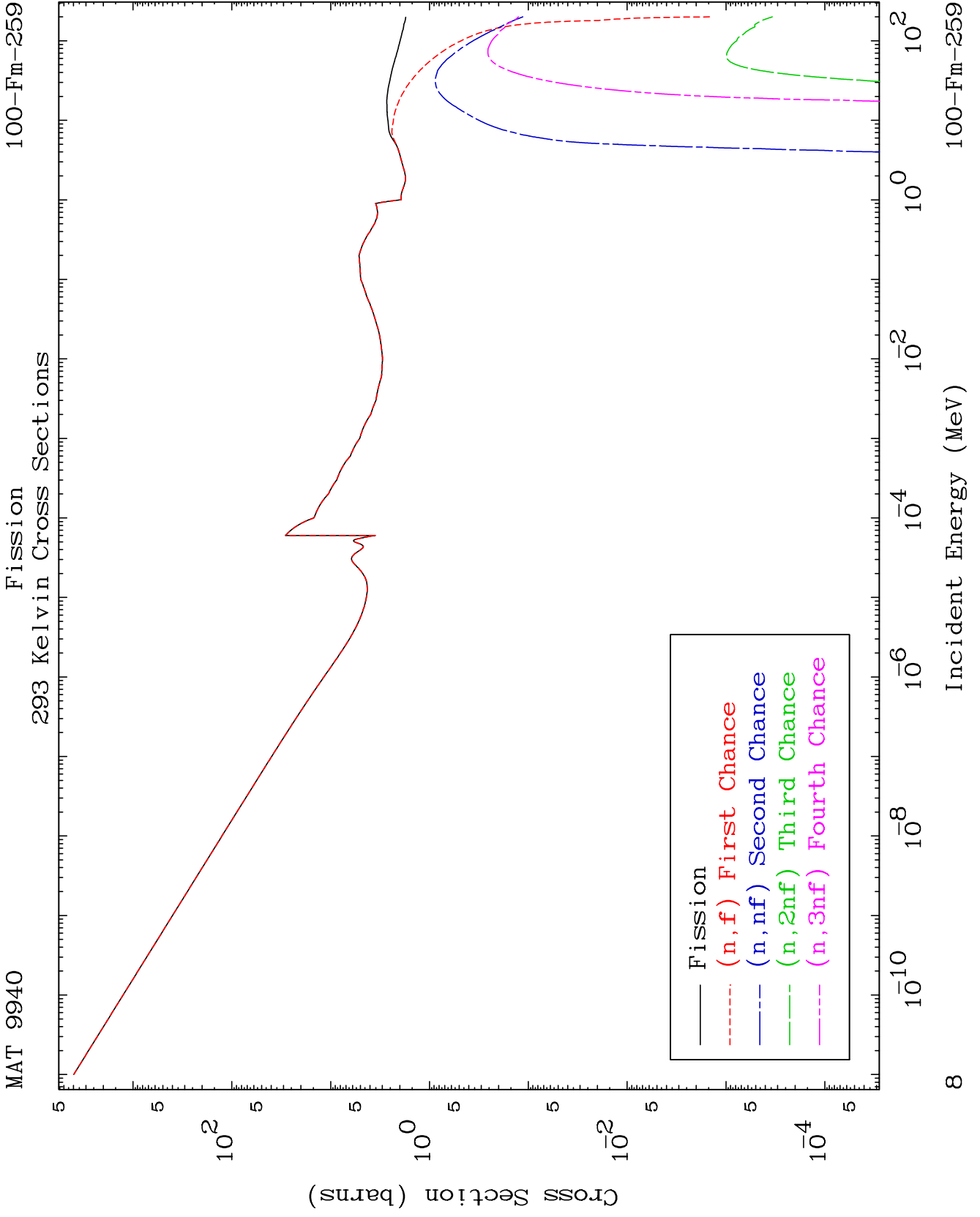


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Particle Production
293 Kelvin Cross Sections

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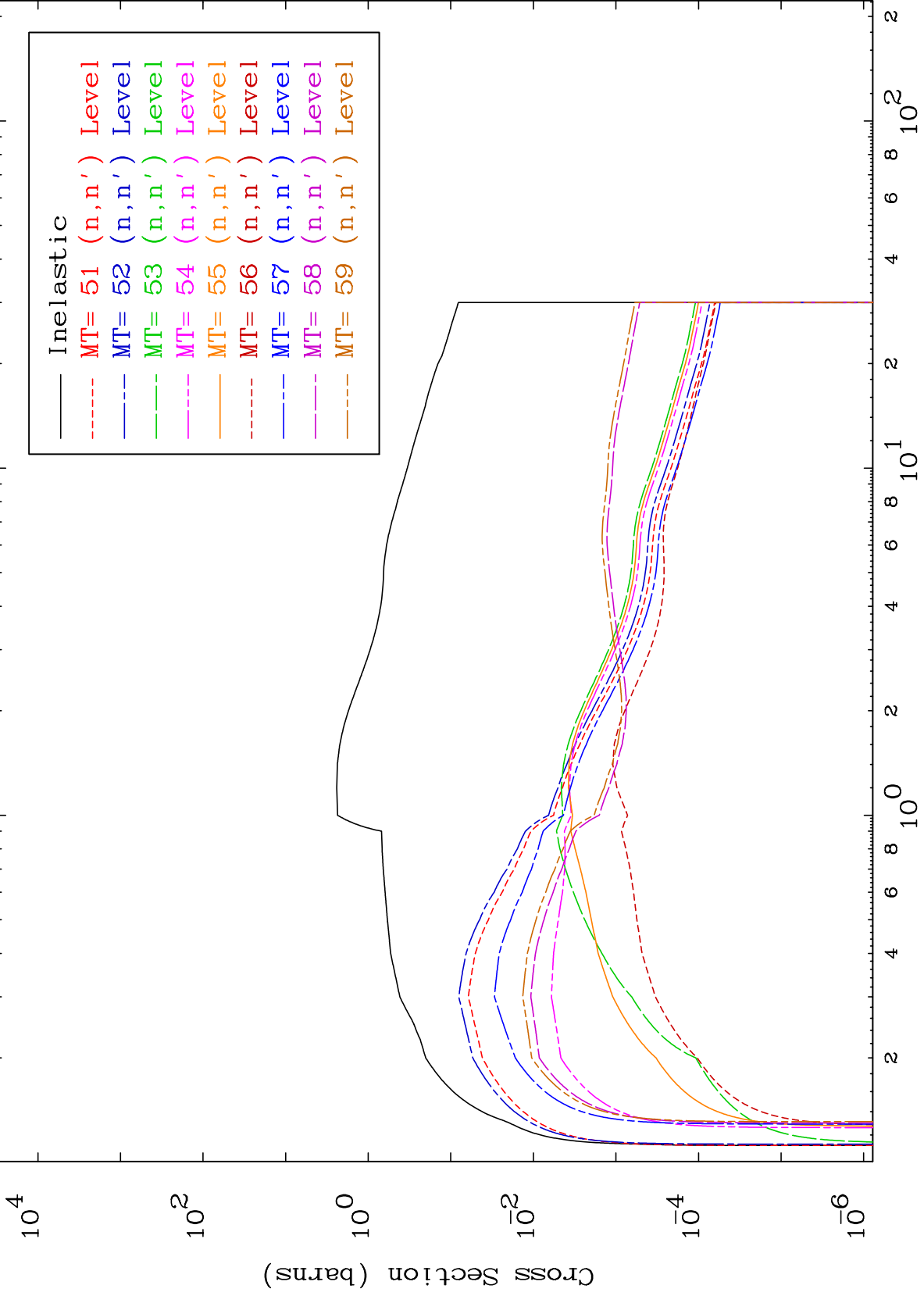


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(n,n') Levels

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293 Kelvin Cross Sections

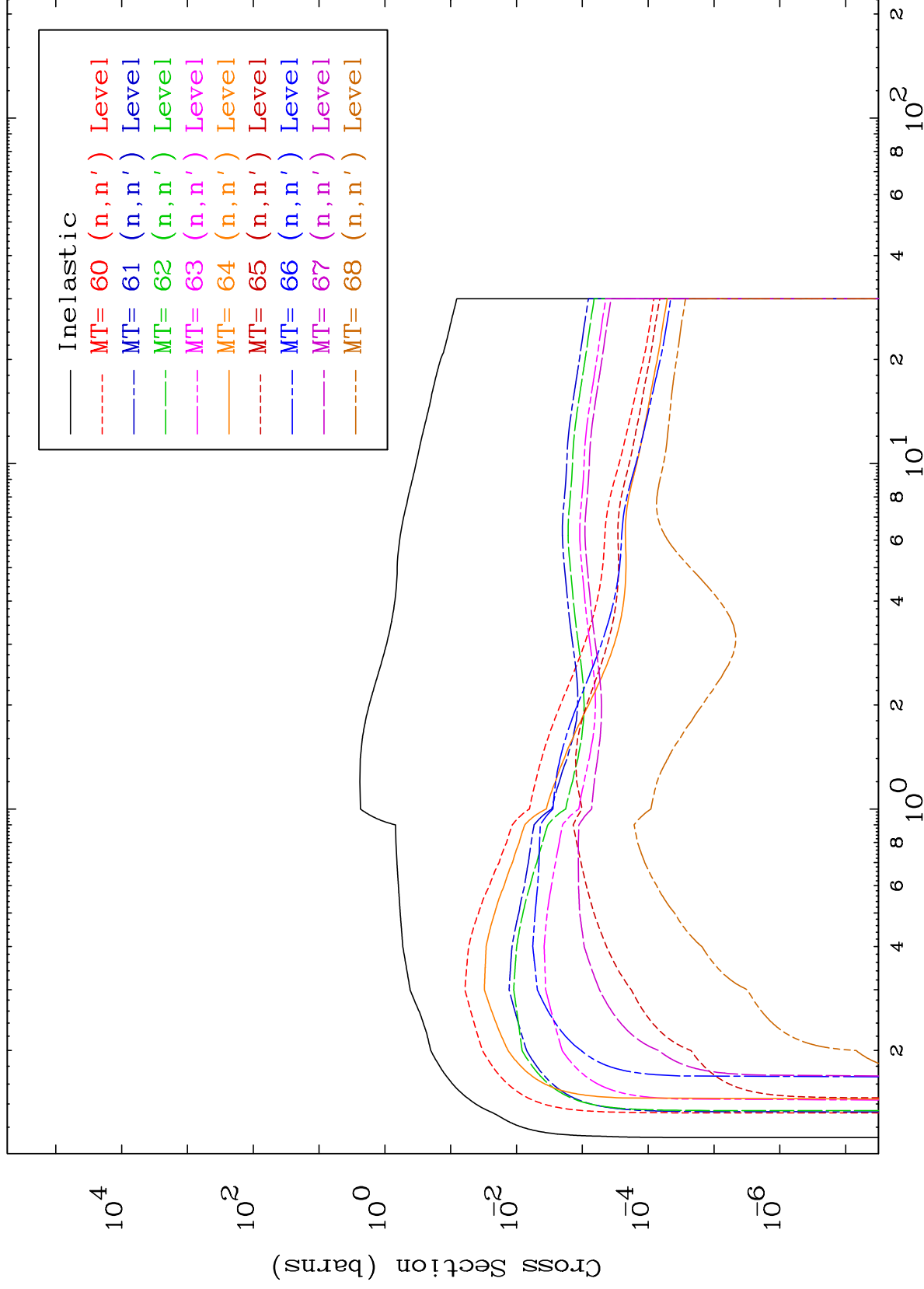


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(n,n') Levels

100-Fm-259

293 Kelvin Cross Sections

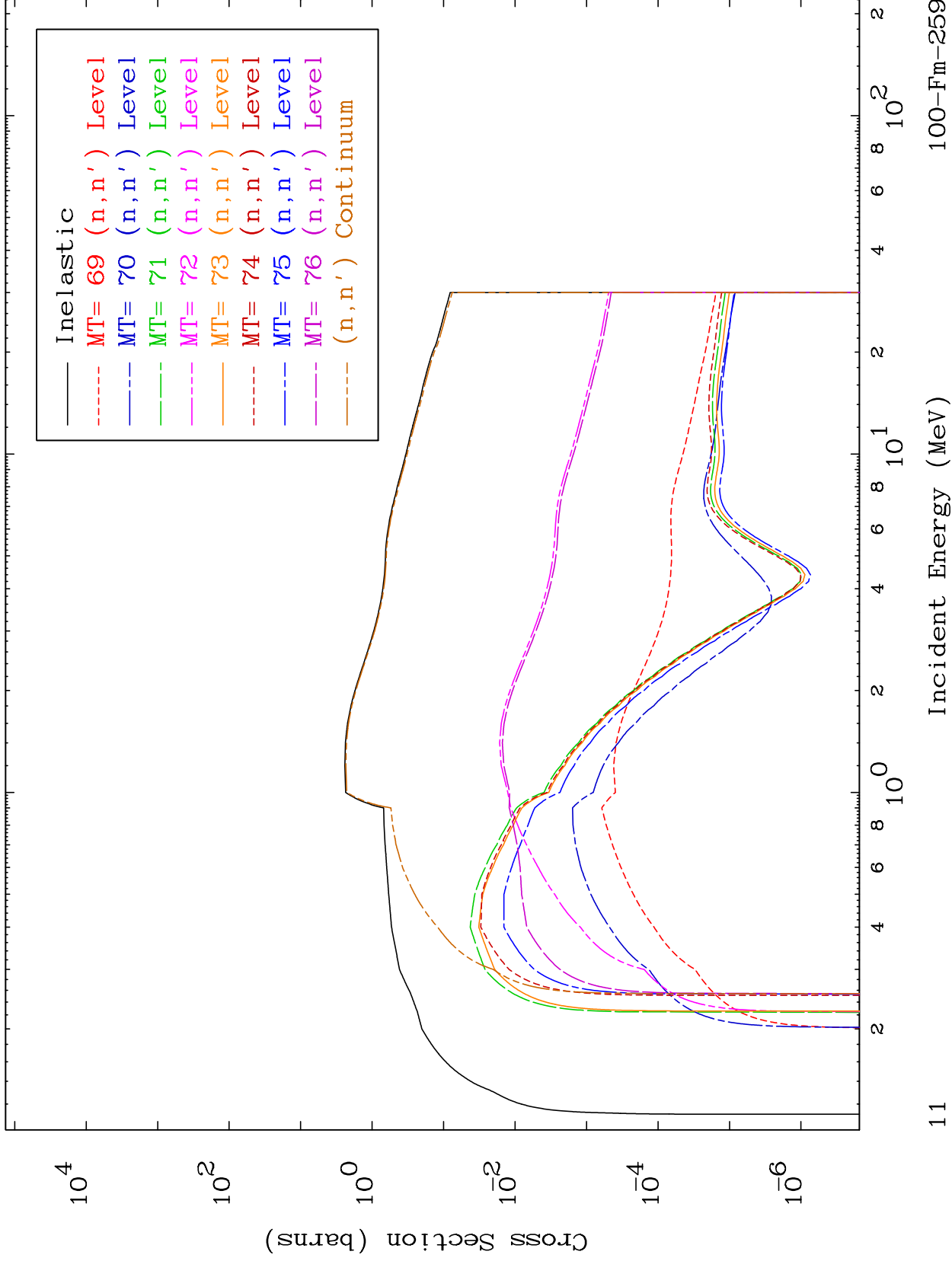


10

Incident Energy (MeV)

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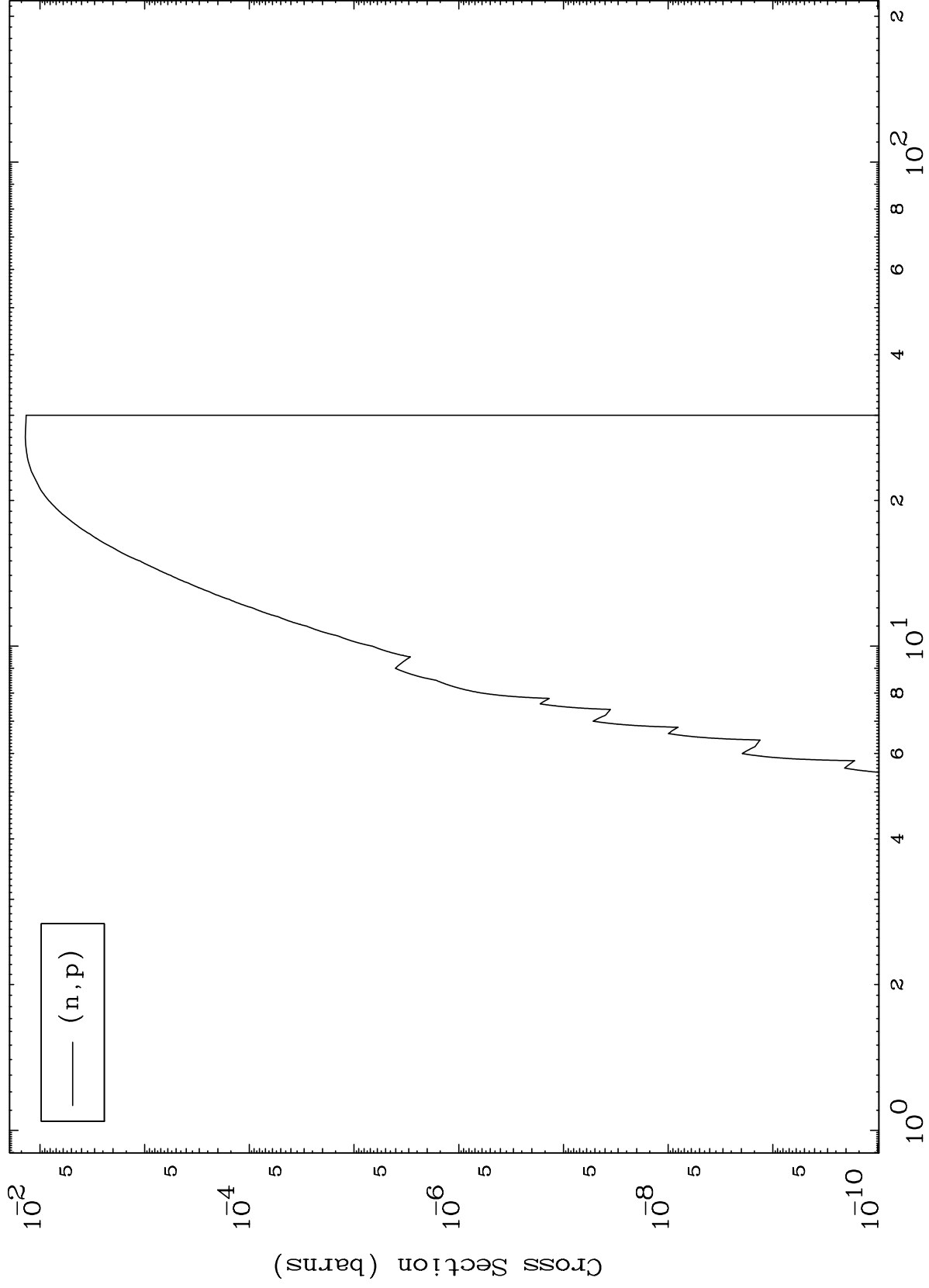
293 Kelvin Cross Sections



MAT 9940

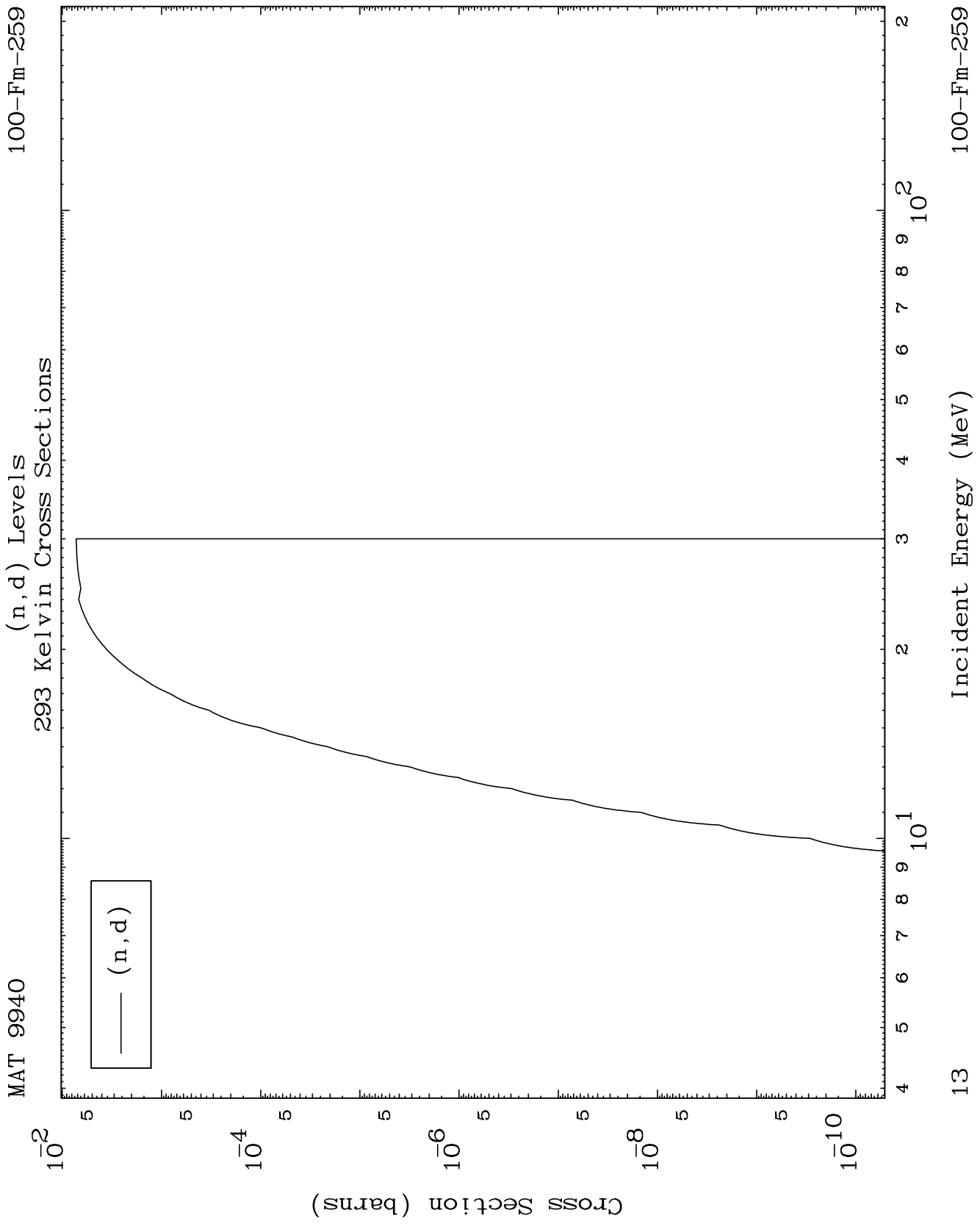
(n,p) Levels
293 Kelvin Cross Sections

100-Fm-259



Incident Energy (MeV)

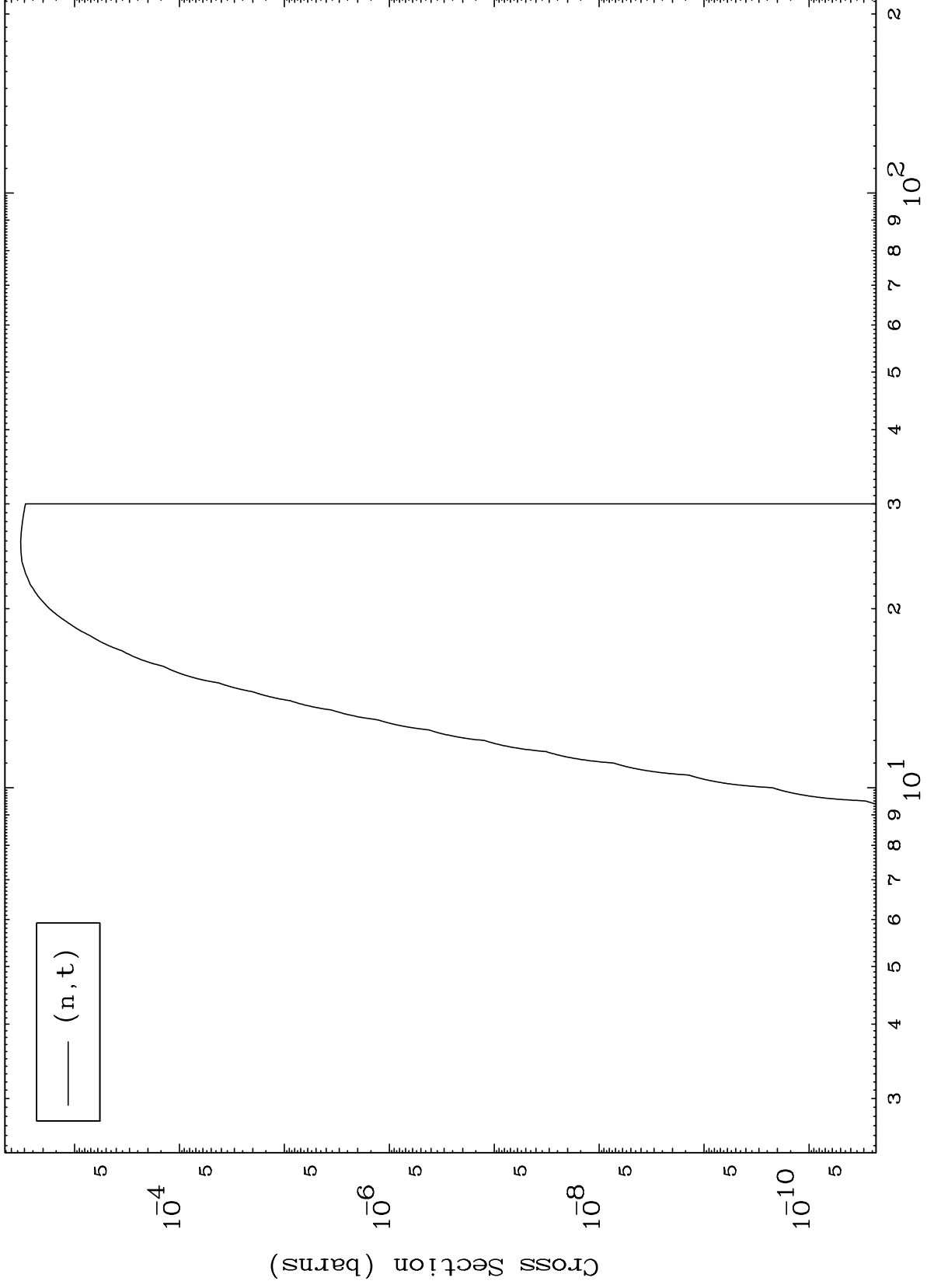
100-Fm-259



MAT 9940

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-259



14

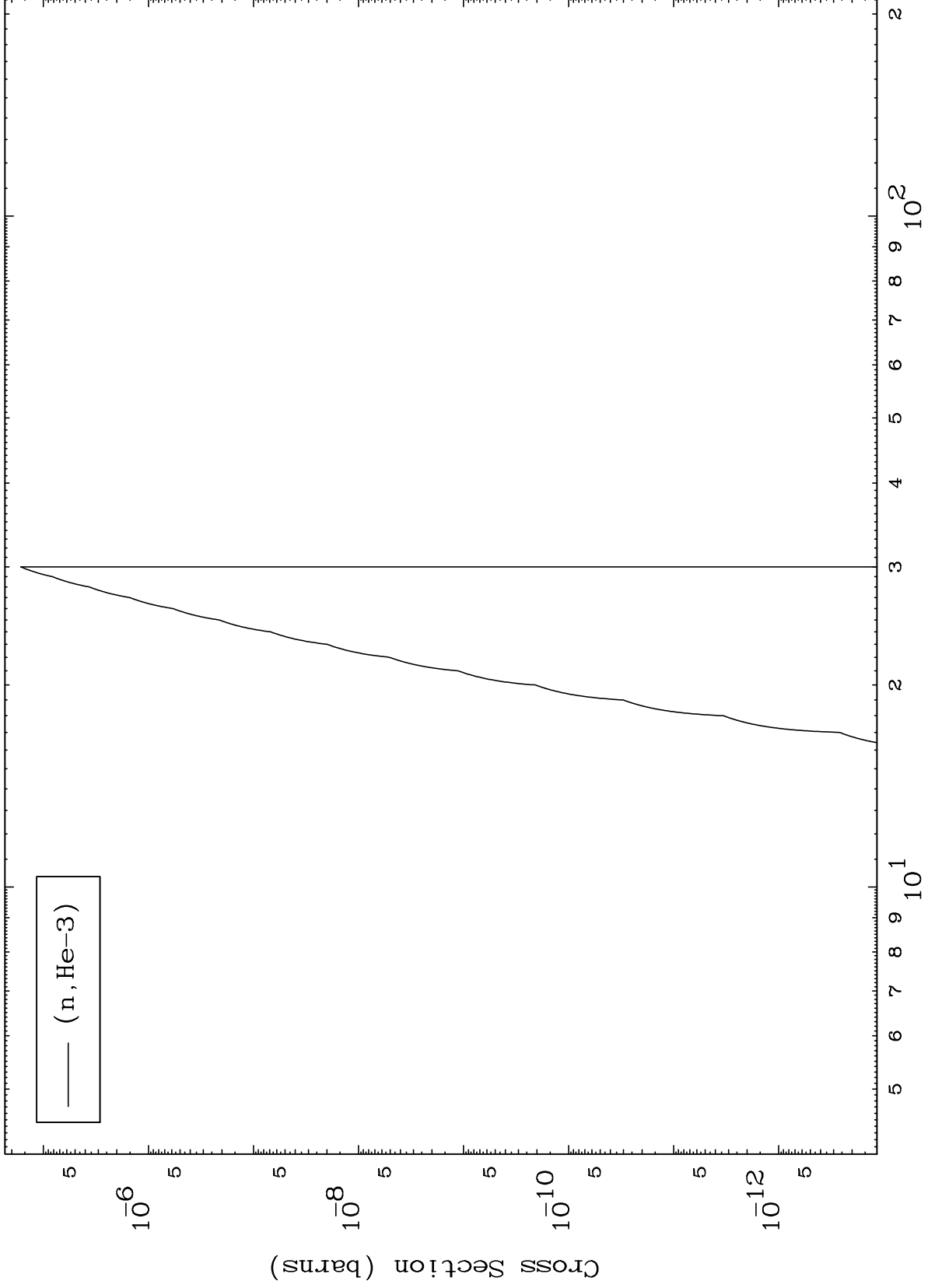
Incident Energy (MeV)

100-Fm-259

MAT 9940

(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-259



15

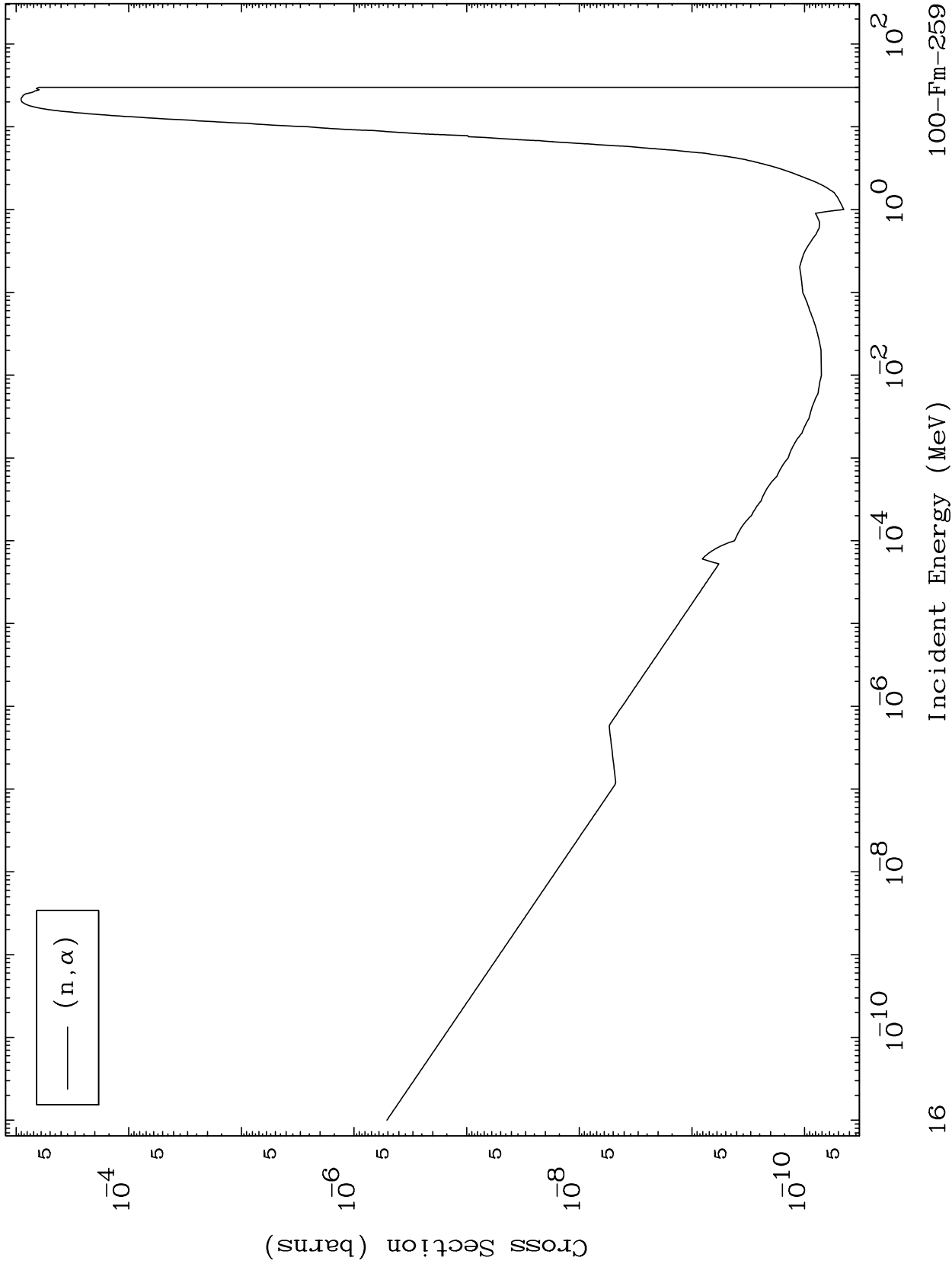
Incident Energy (MeV)

100-Fm-259

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(n, α) Levels
293 Kelvin Cross Sections

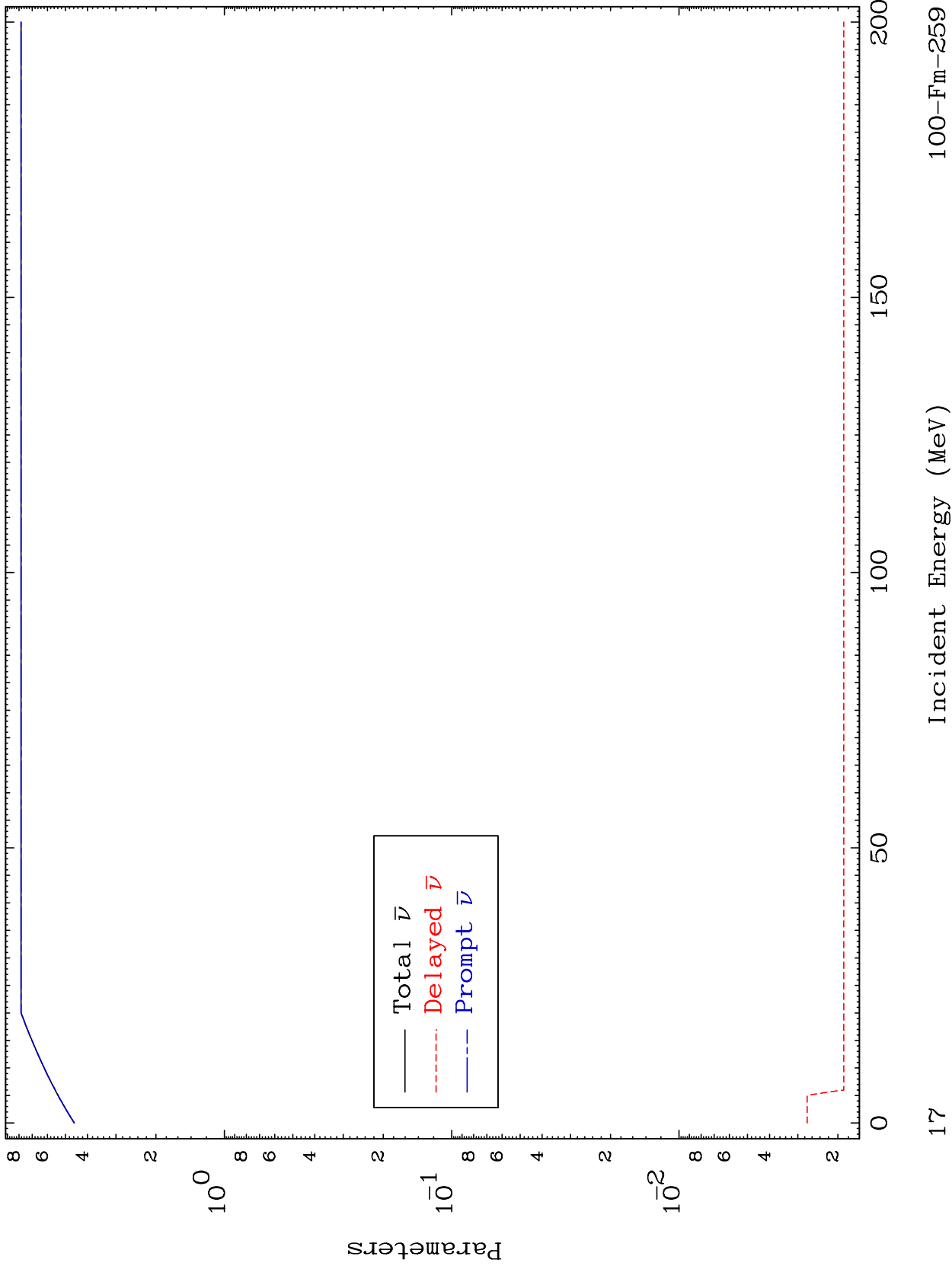
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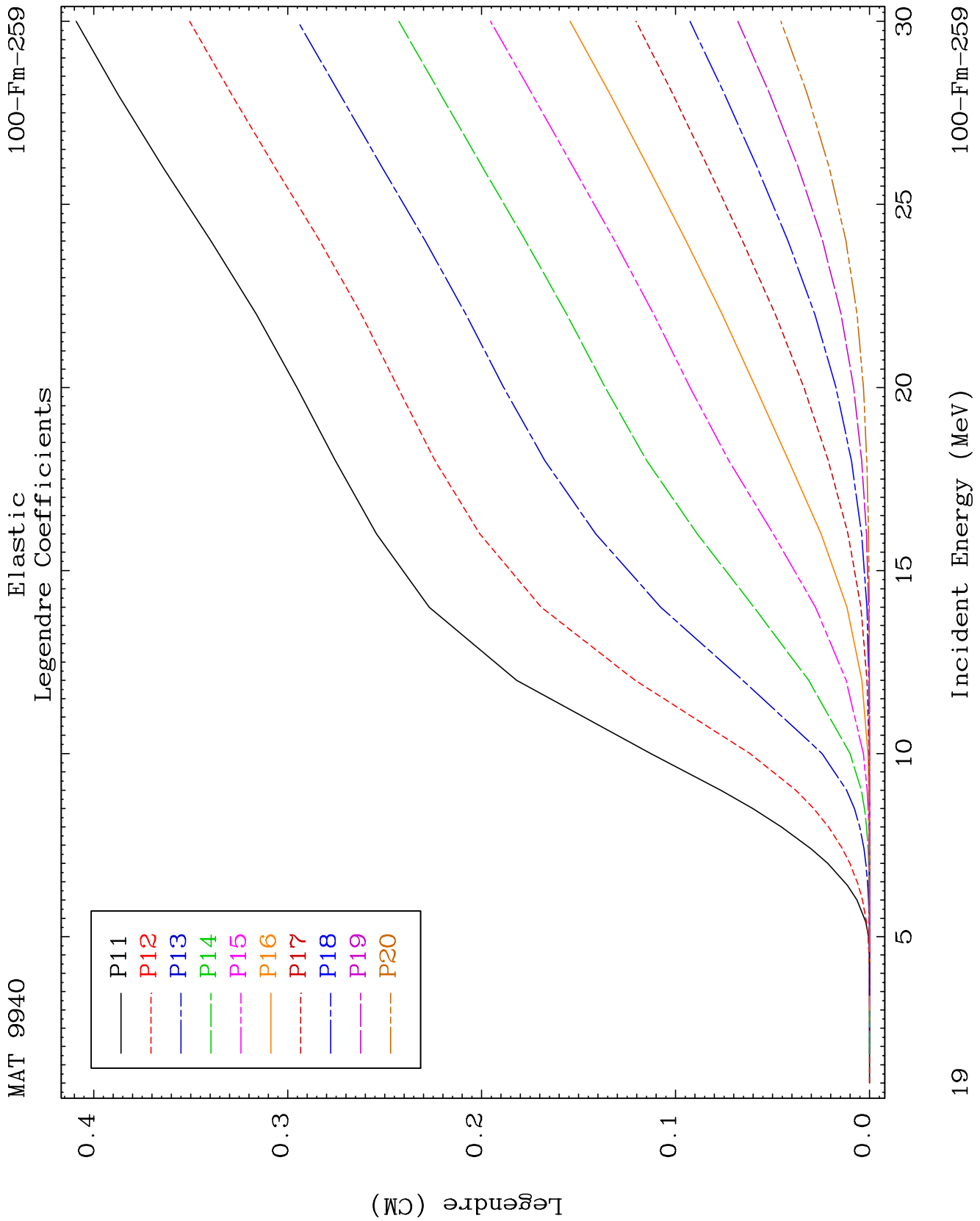


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Energy Release
Parameters

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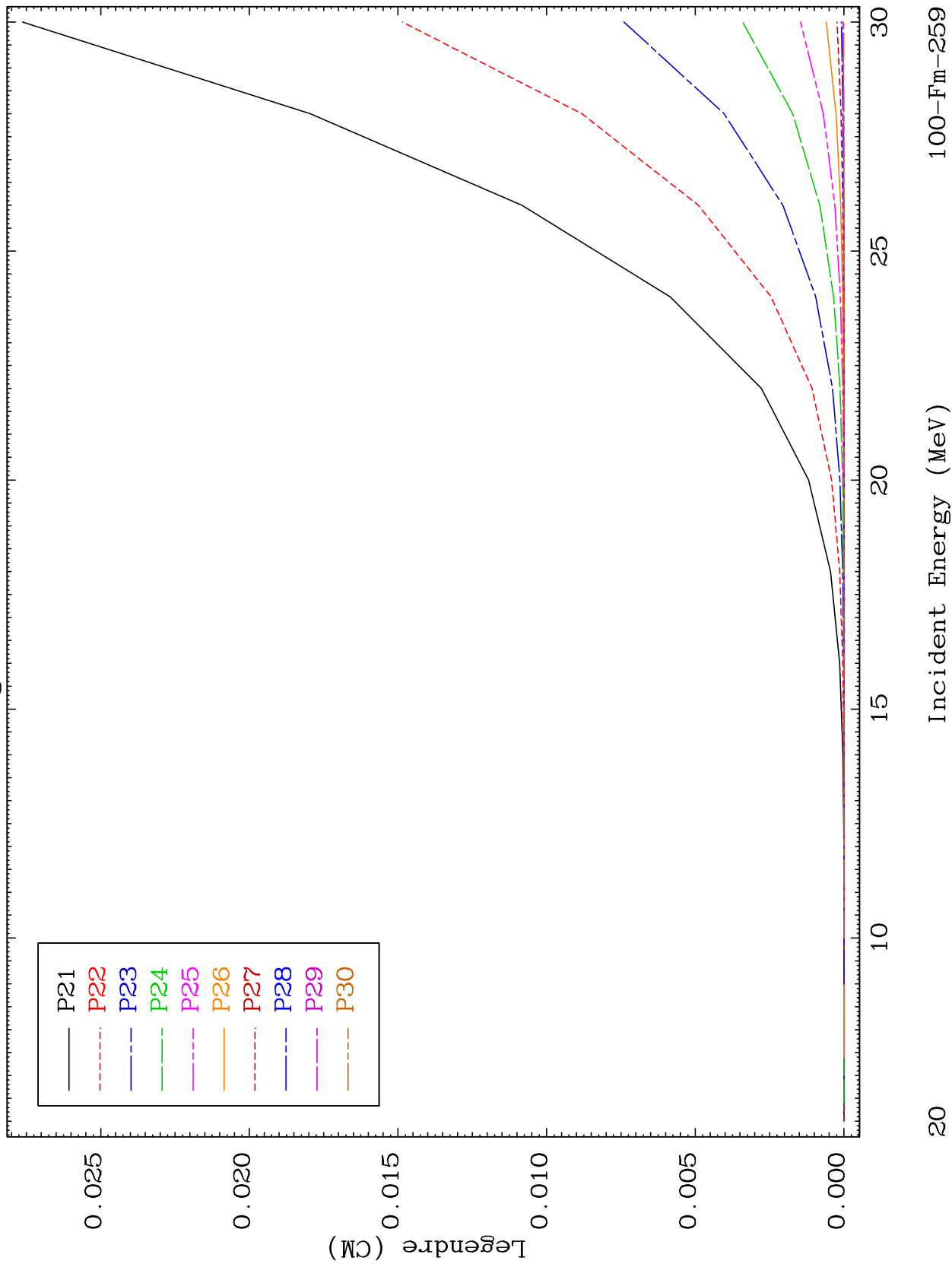




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Elastic Legendre Coefficients

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20

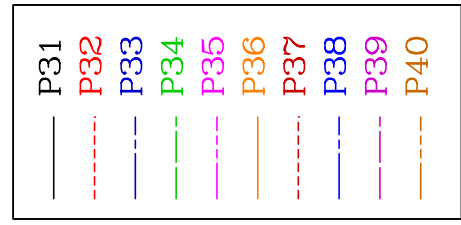
Incident Energy (MeV)

100-Fm-259

MAT 9940

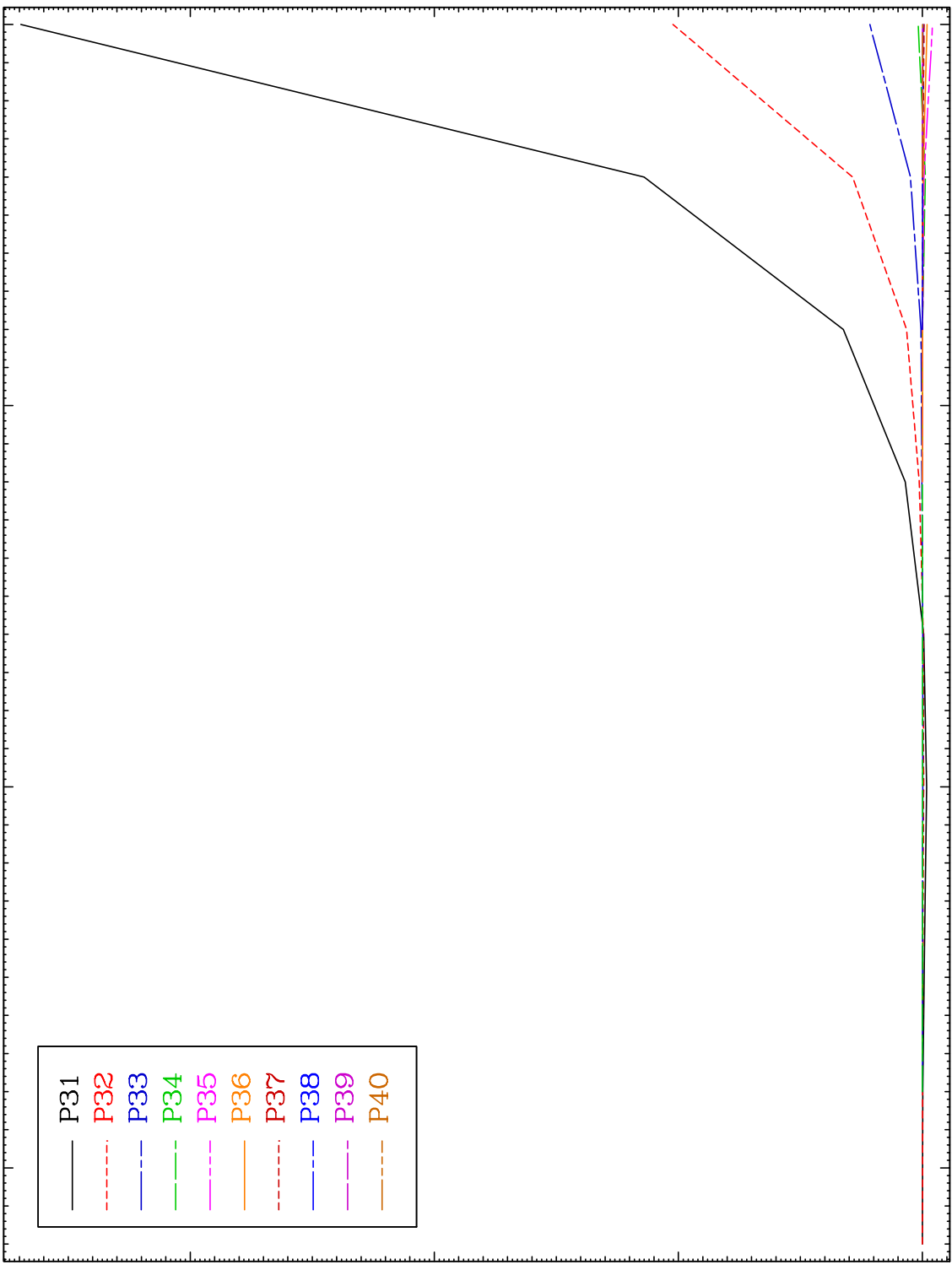
Elastic Legendre Coefficients

100-Fm-259



$\times 10^{-6}$

Legendre (CM)



30

25

20

15

21

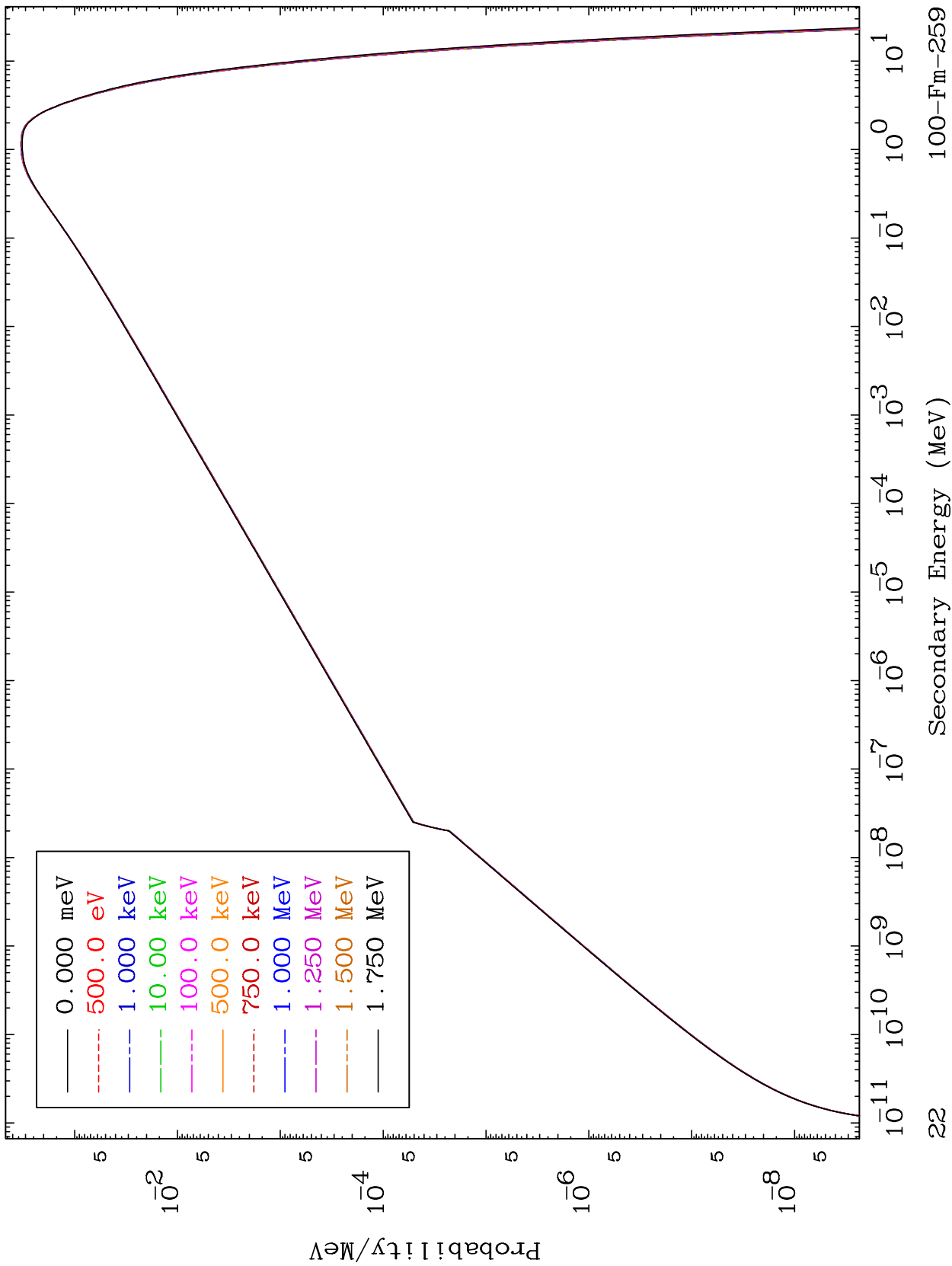
Incident Energy (MeV)

100-Fm-259

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Fission Energy Distributions

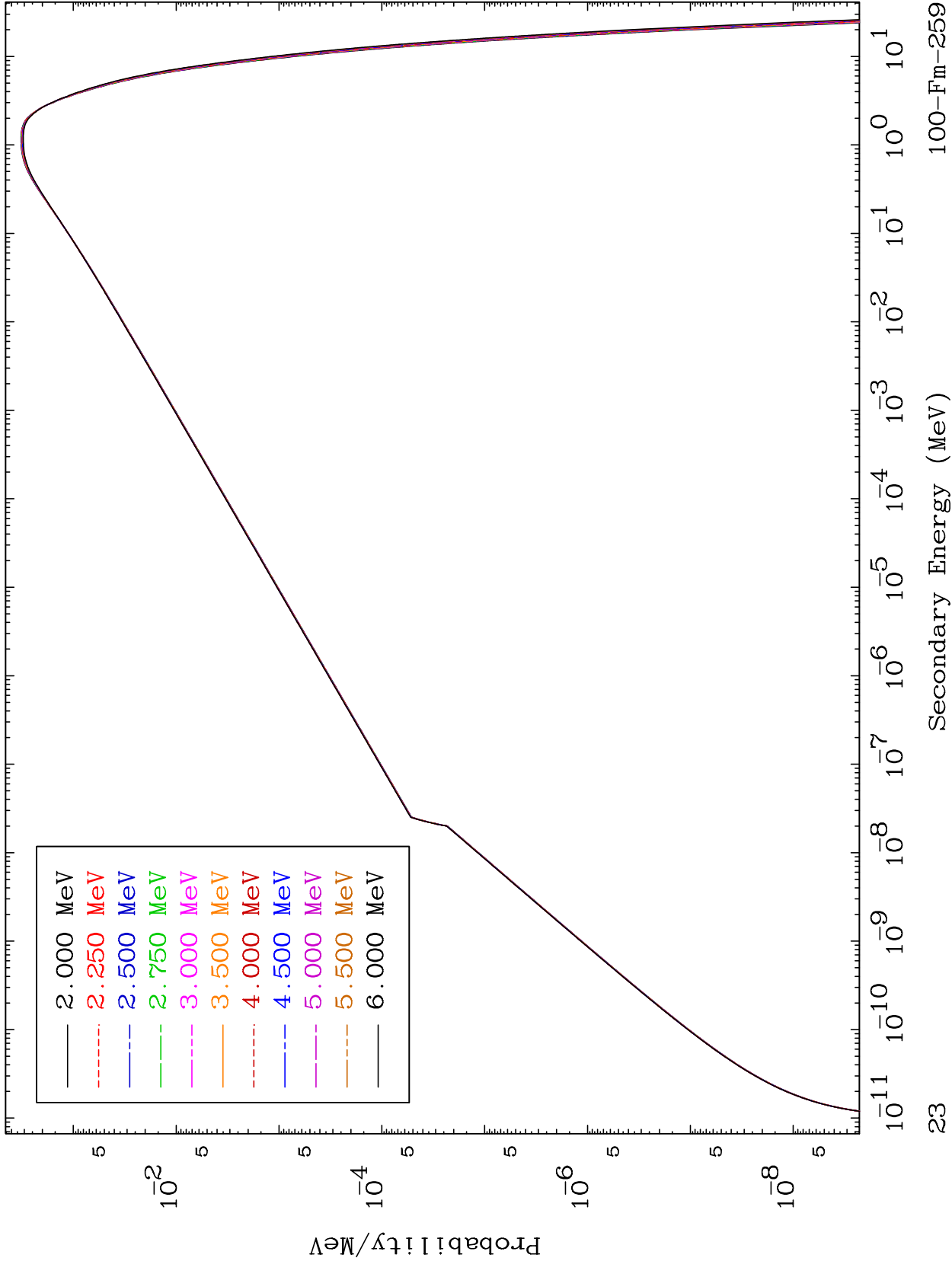
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Fission Energy Distributions

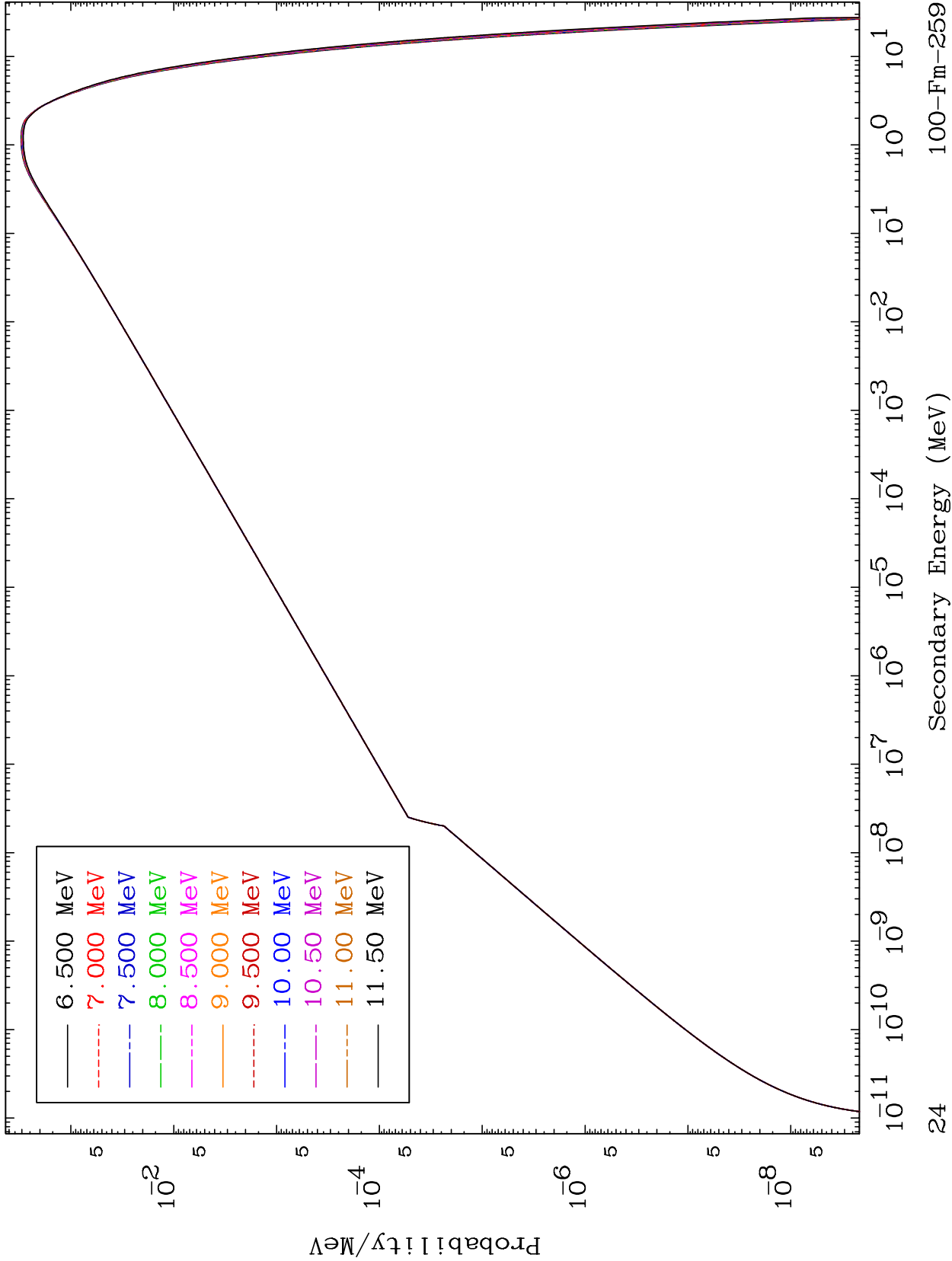
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Fission Energy Distributions

100-Fm-259



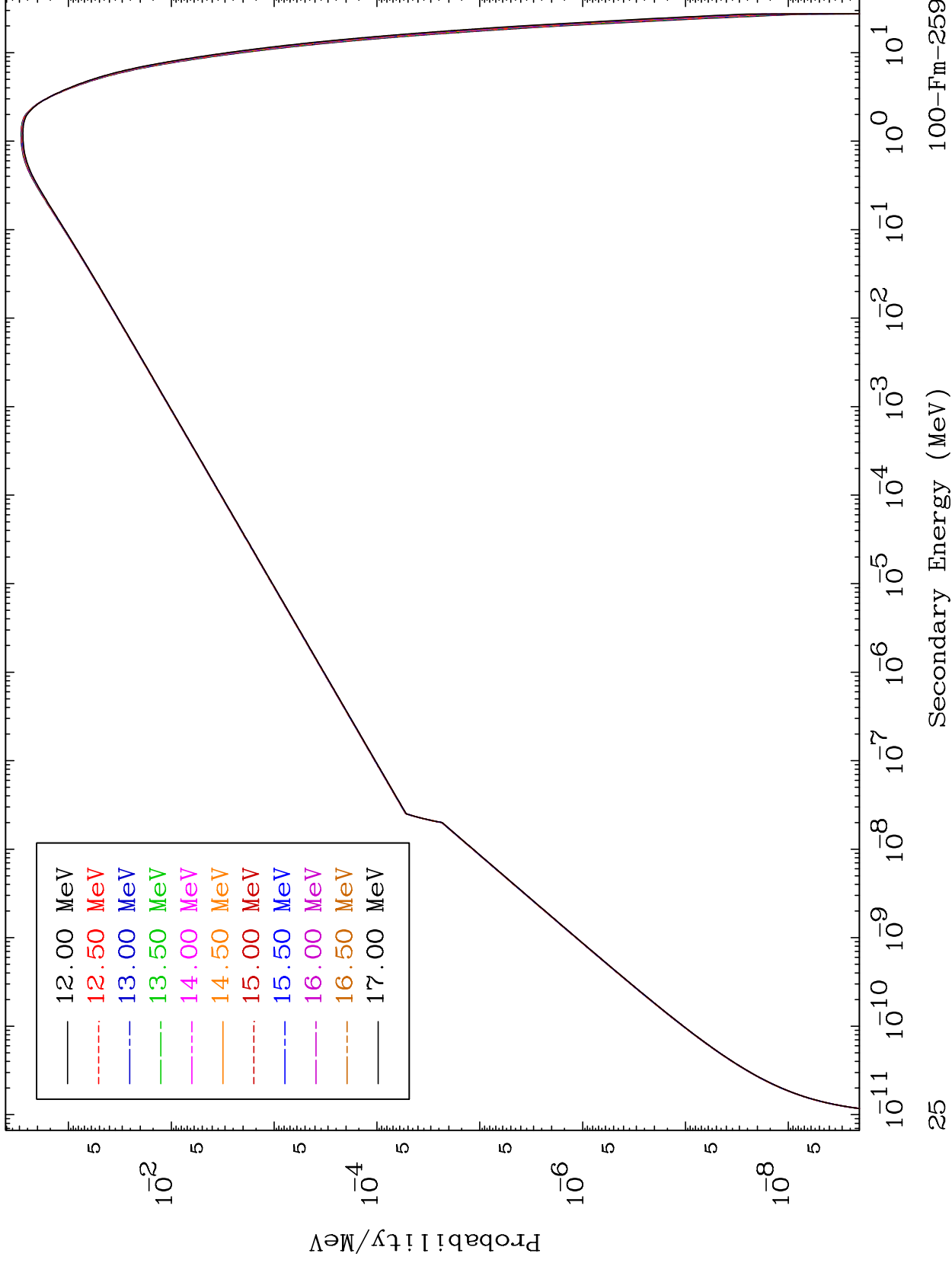
100-Fm-259

Secondary Energy (MeV)

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Fission Energy Distributions

100-Fm-259



100-Fm-259

Secondary Energy (MeV)

25

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Fission Energy Distributions

100-Fm-259

