

Program EVALPLOT  
(Version 2021-1)

by

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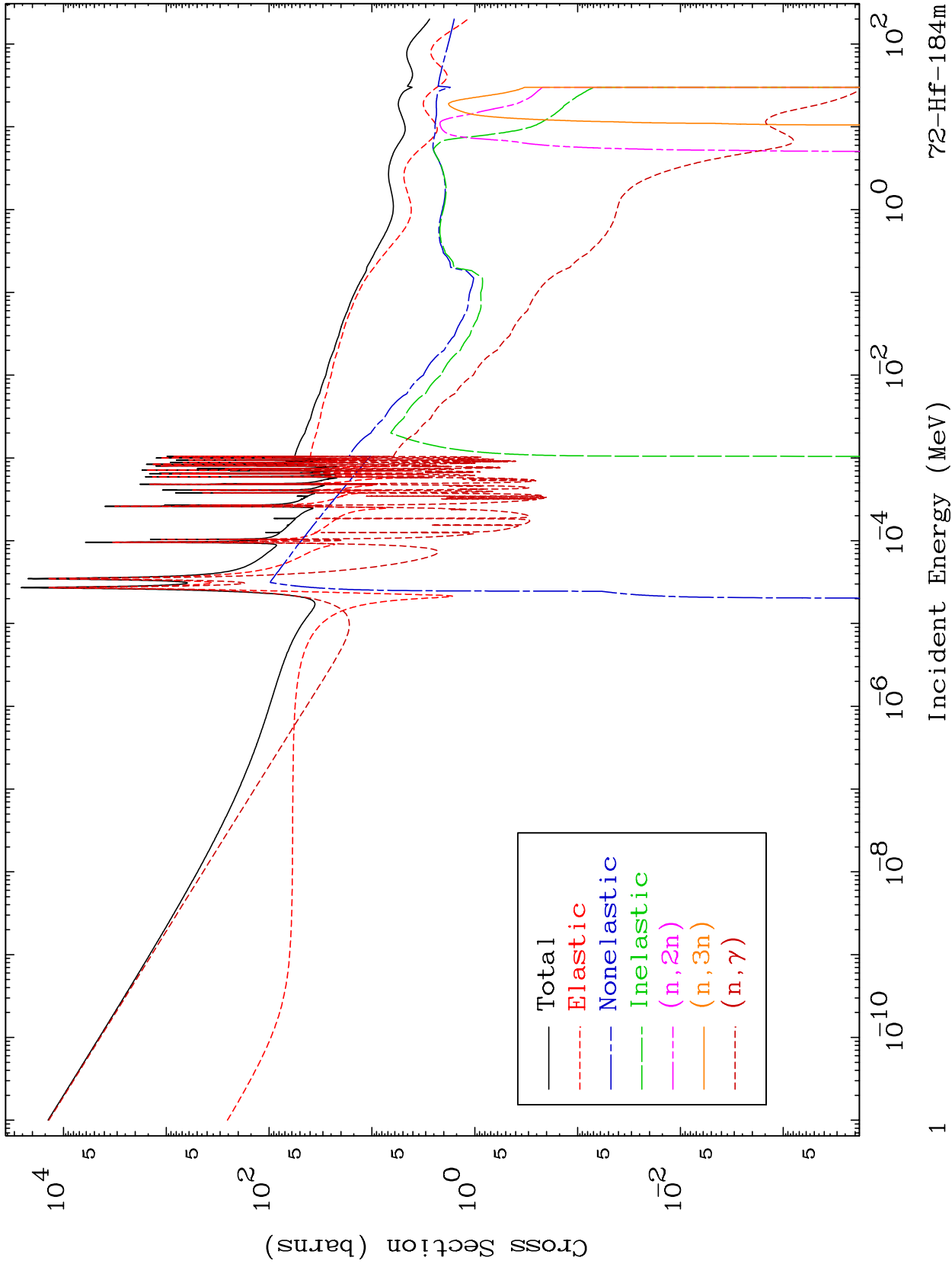
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7256

Neutron Major  
293 Kelvin Cross Sections

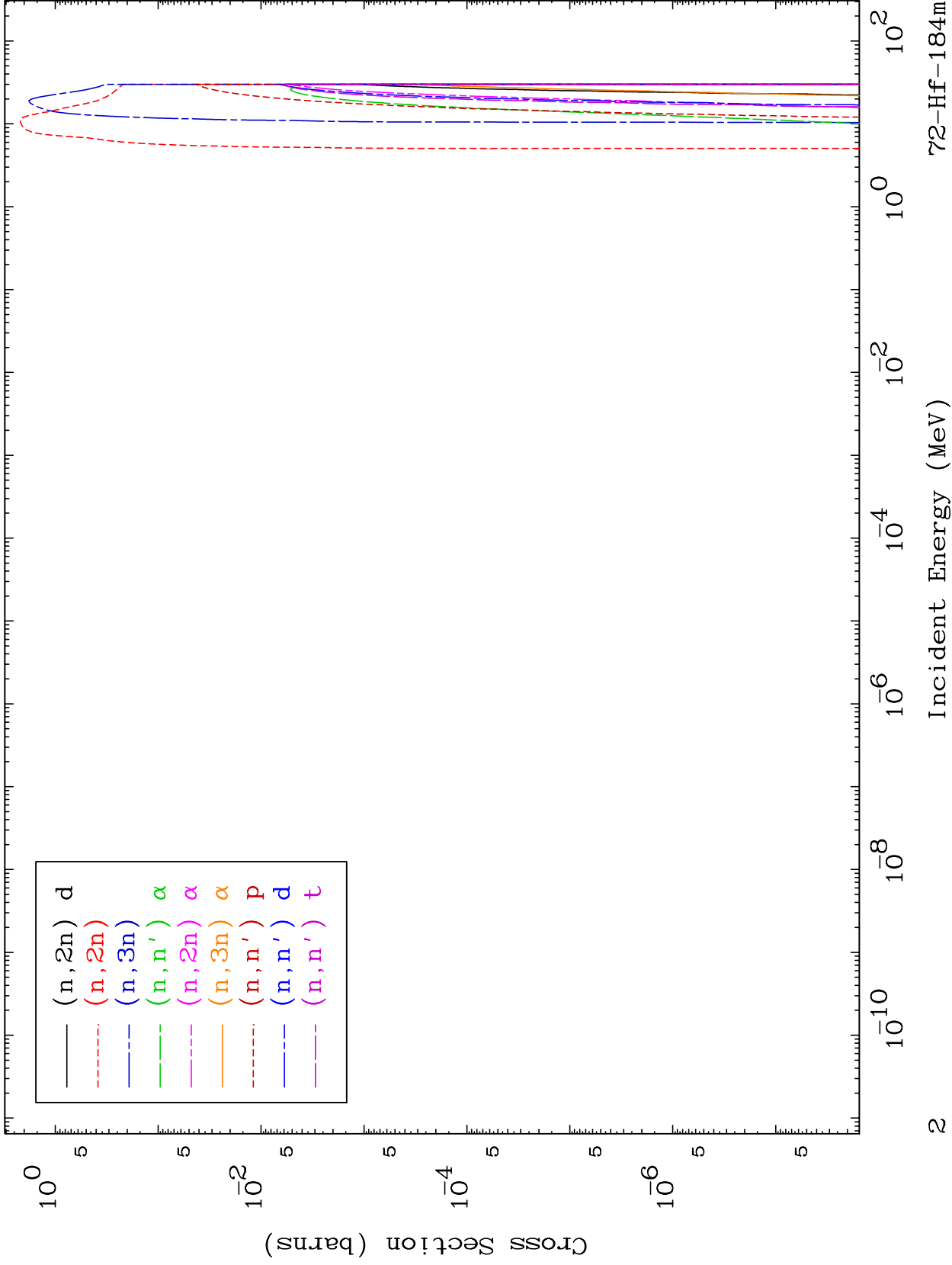
72-Hf-184m



MAT 7256

Neutron Absorption  
293 Kelvin Cross Sections

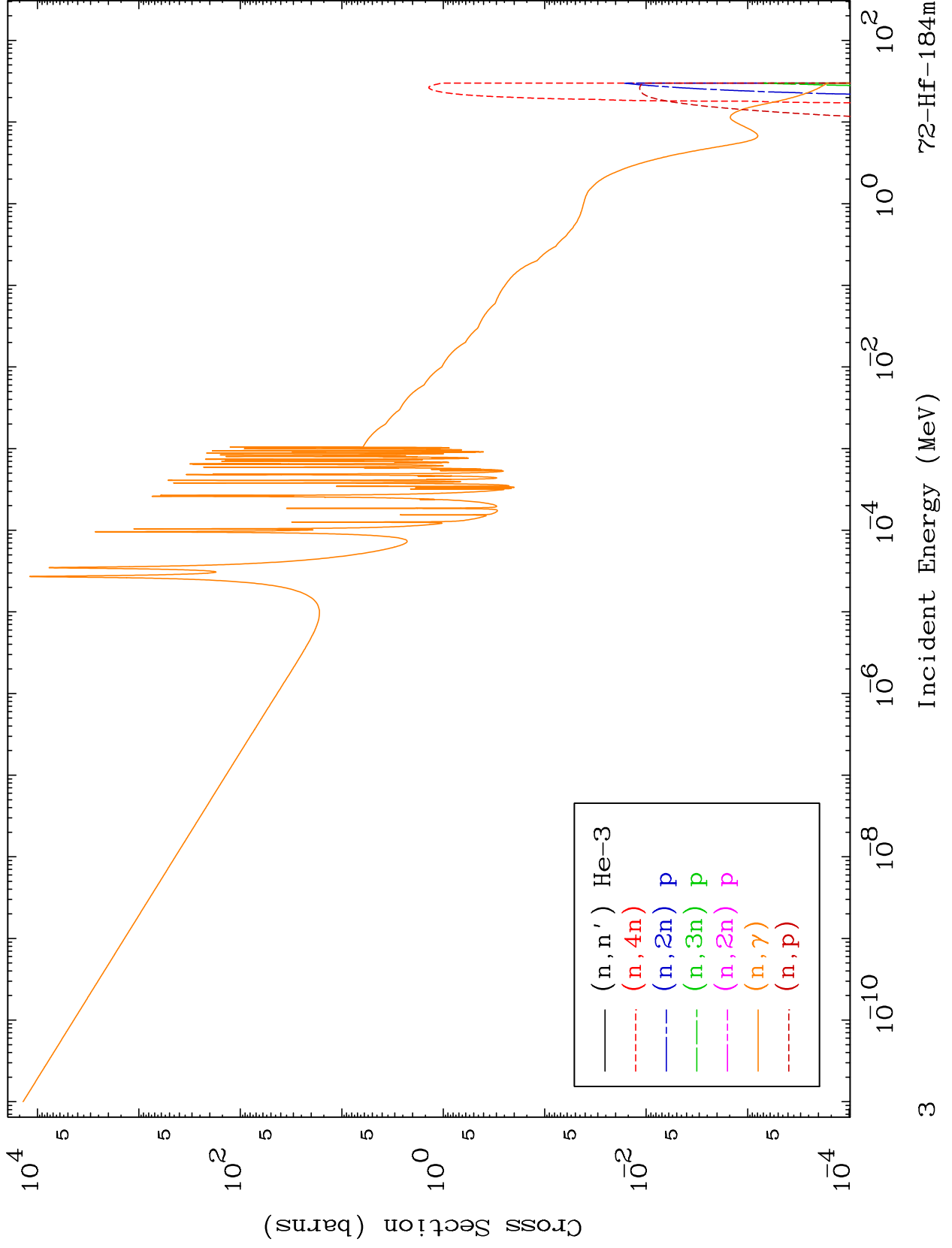
72-Hf-184m



MAT 7256

Neutron Absorption  
293 Kelvin Cross Sections

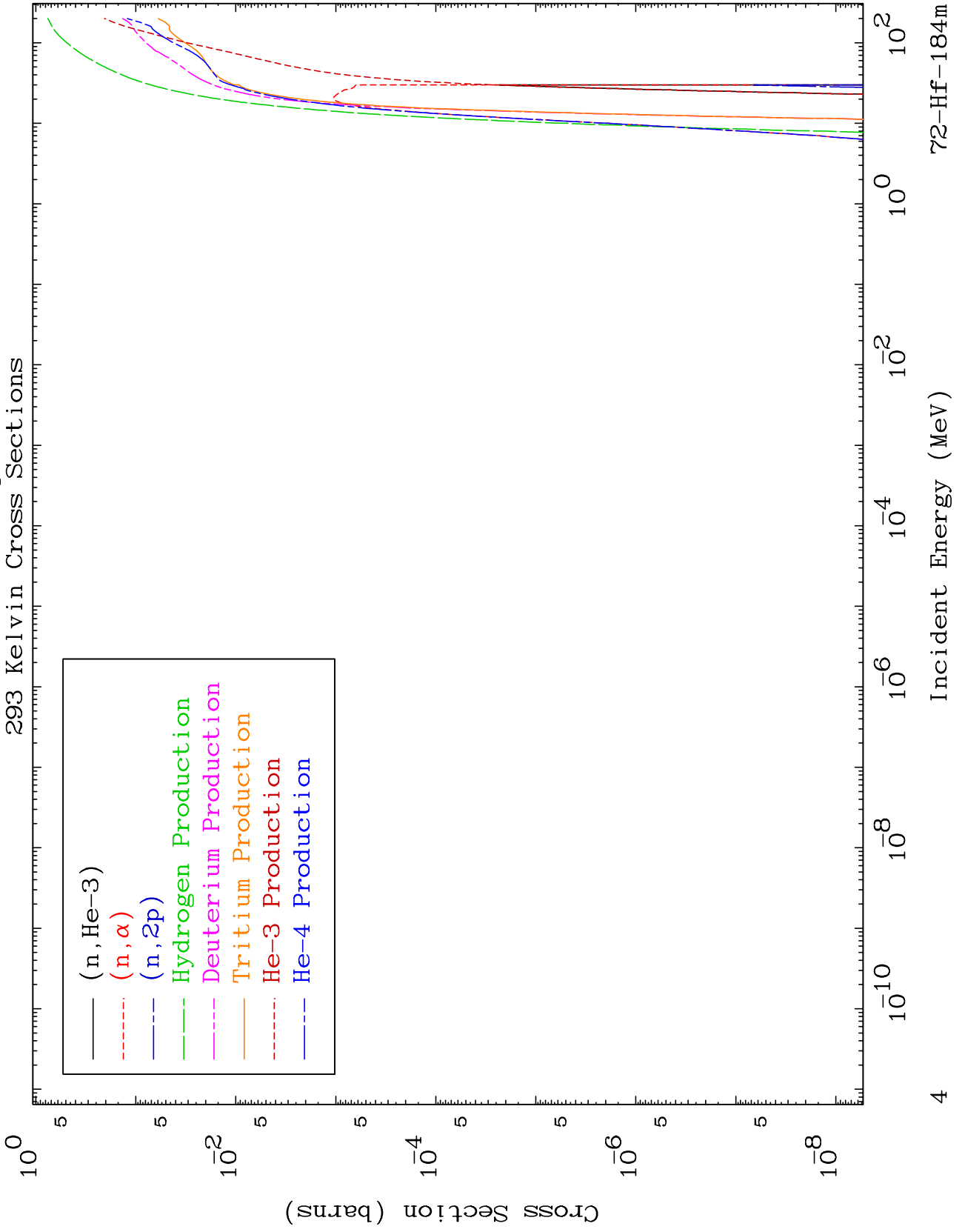
72-Hf-184m



MAT 7256

Neutron Absorption  
293 Kelvin Cross Sections

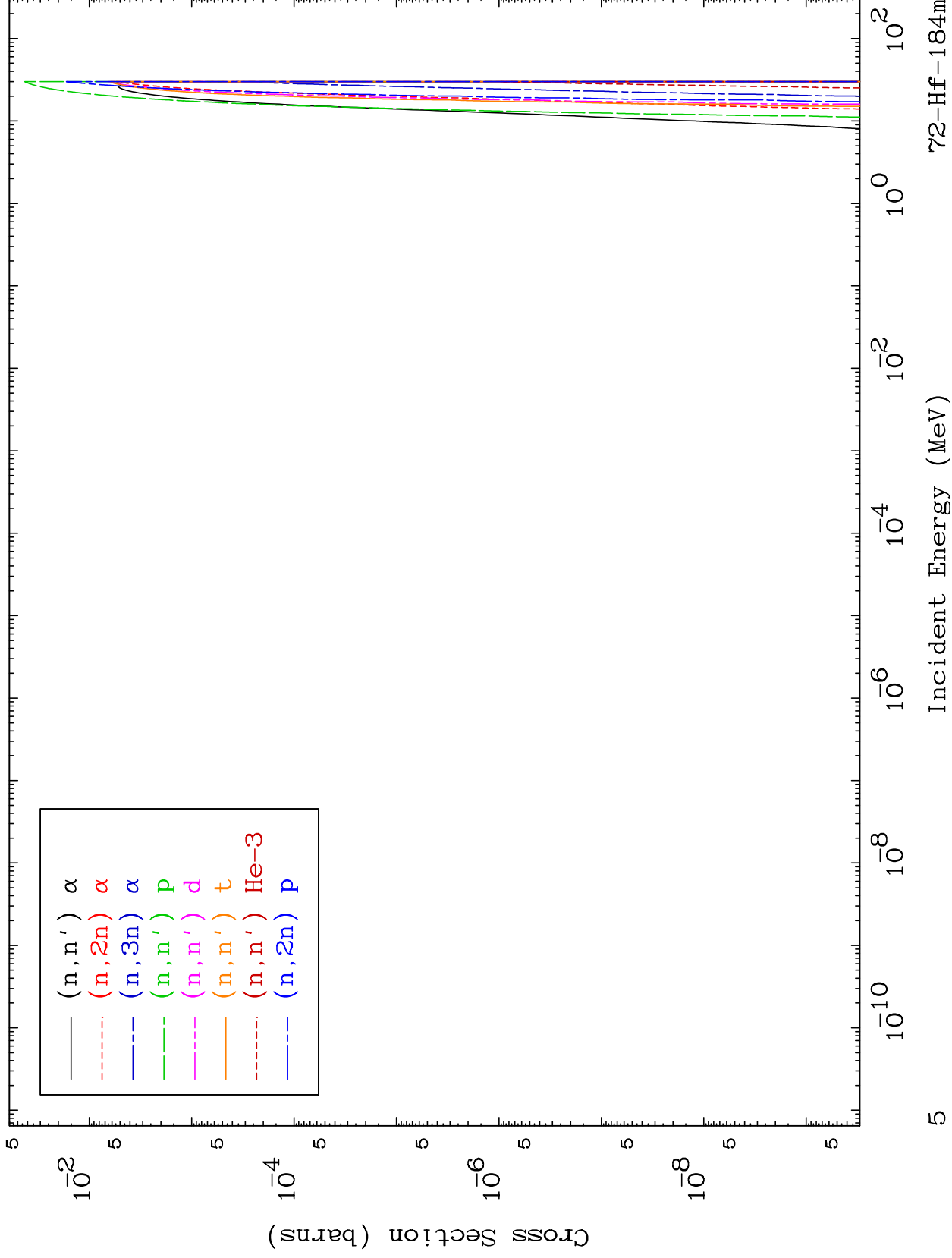
<sup>72</sup>Hf-184m



MAT 7256

Charged Particle  
293 Kelvin Cross Sections

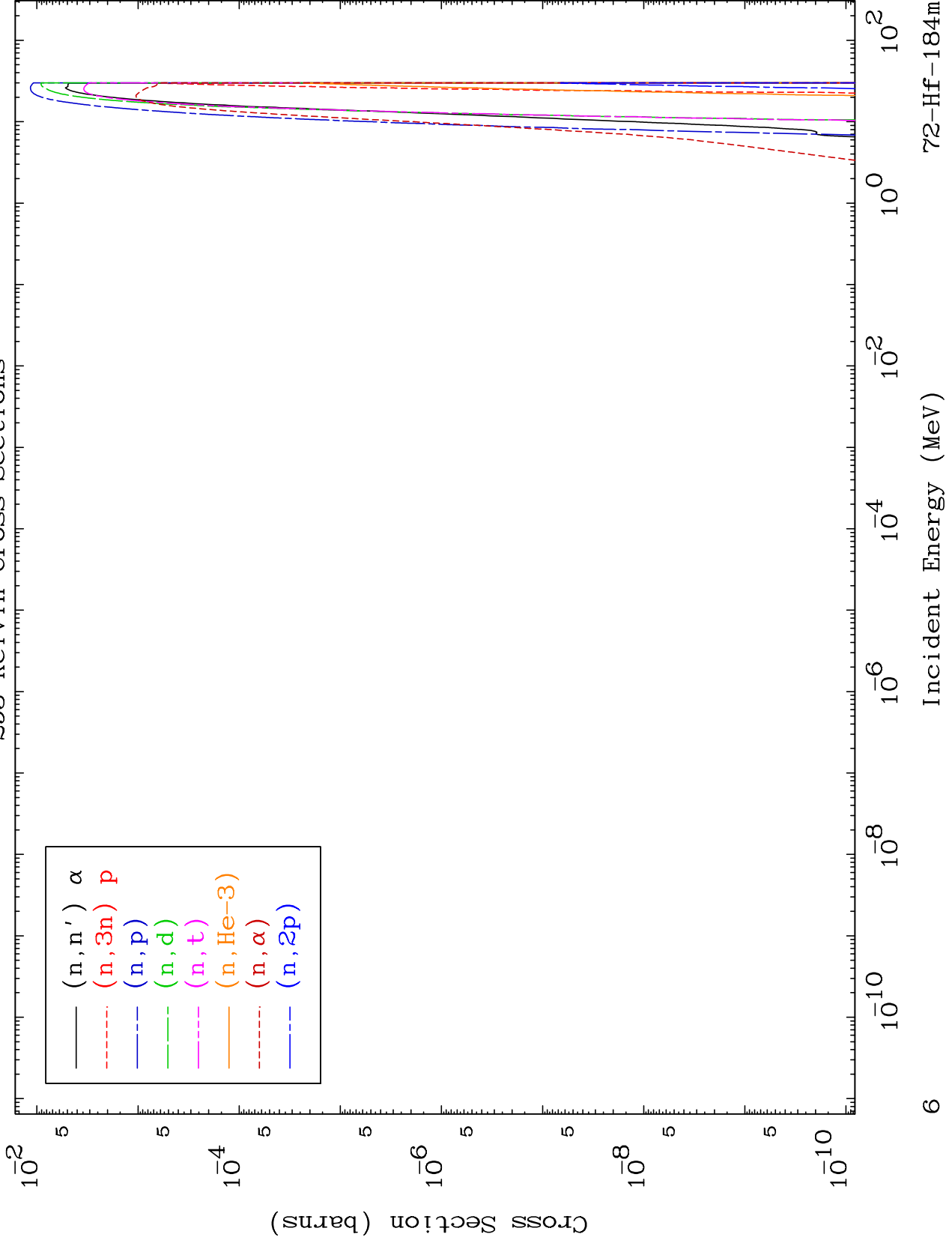
72-Hf-184m



MAT 7256

Charged Particle  
293 Kelvin Cross Sections

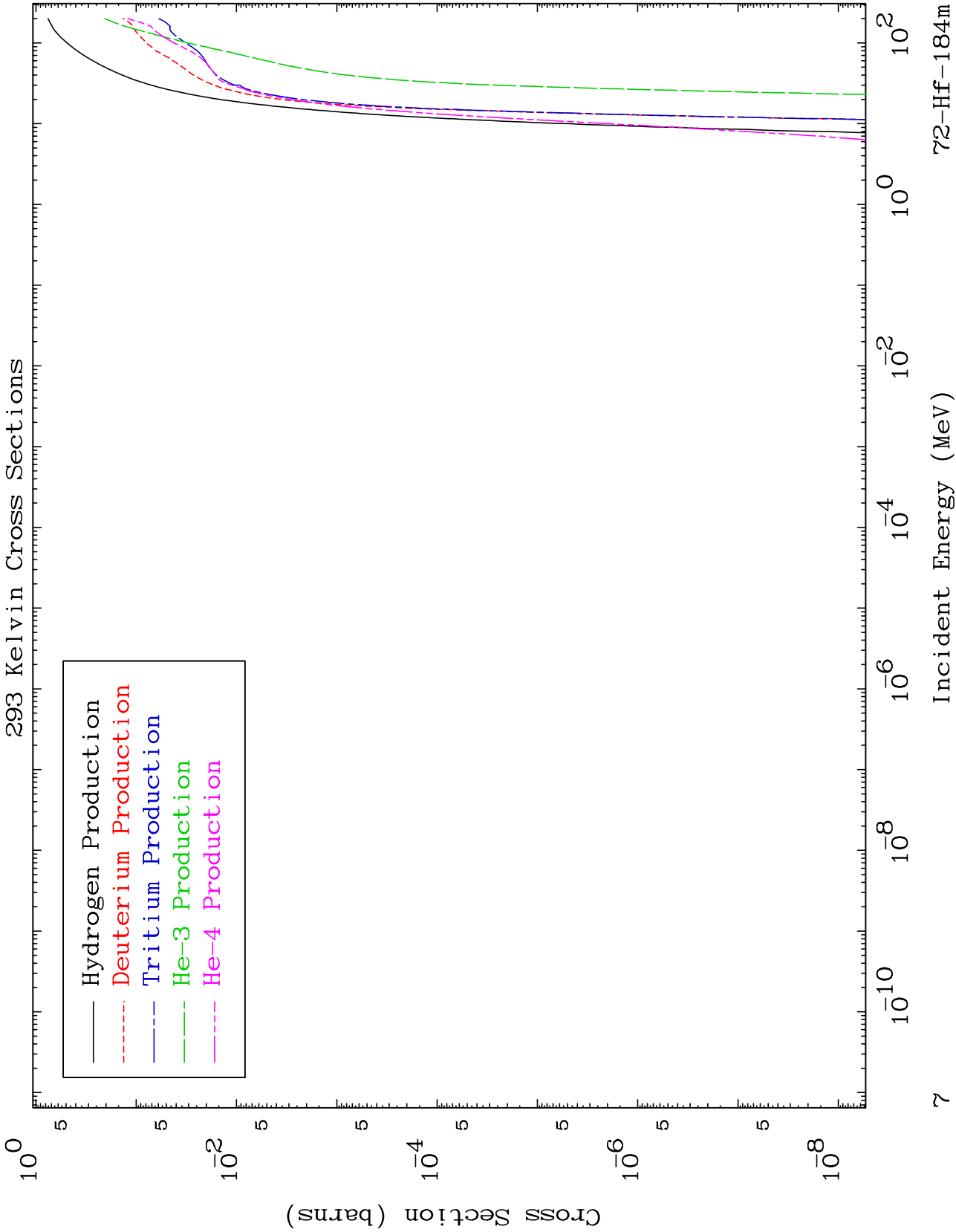
72-Hf-184m



MAT 7256

Particle Production  
293 Kelvin Cross Sections

72-Hf-184m

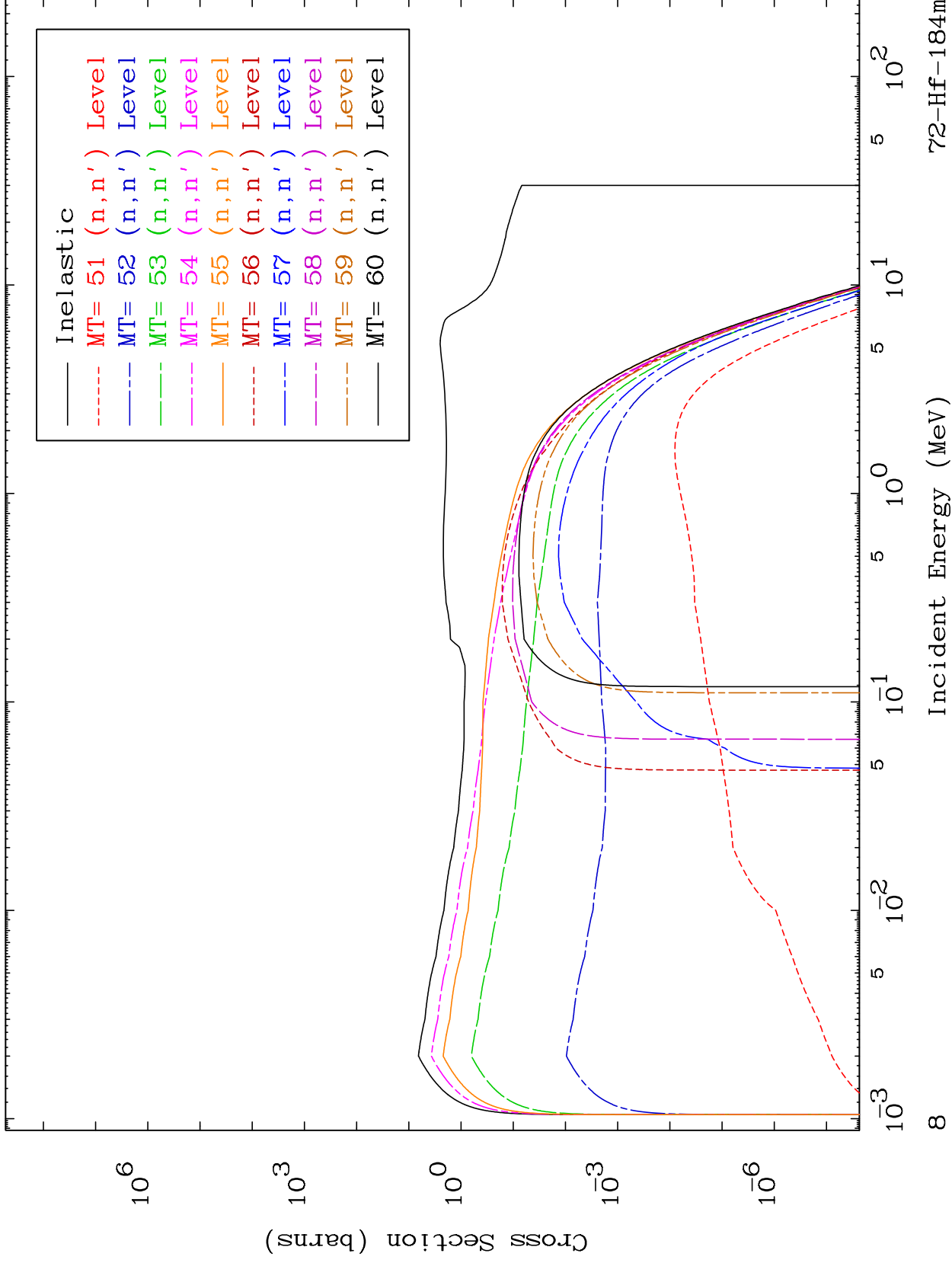


MAT 7256

(n,n') Levels

72-Hf-184m

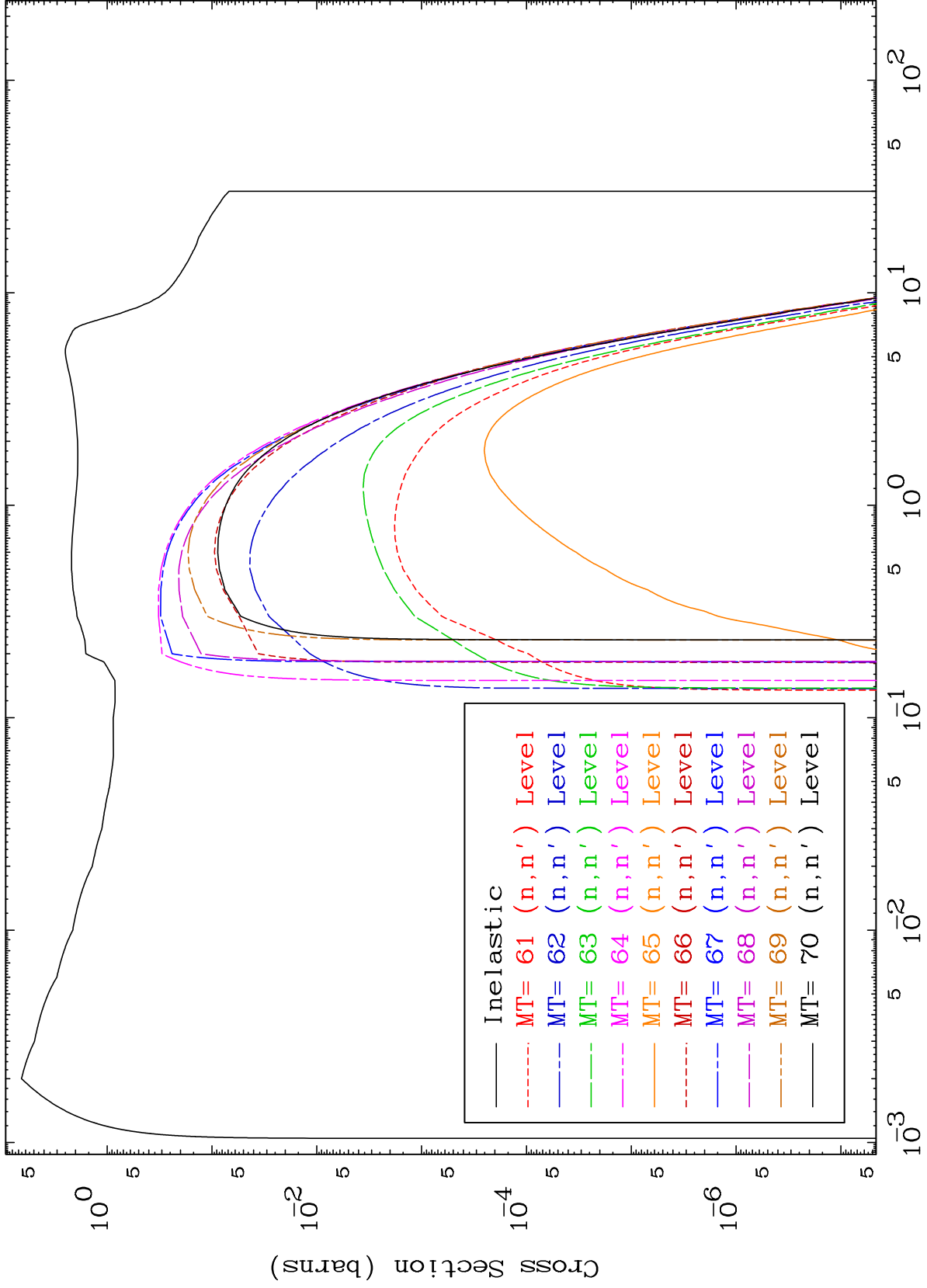
293 Kelvin Cross Sections



MAT 7256

(n,n') Levels  
293 Kelvin Cross Sections

72-Hf-184m



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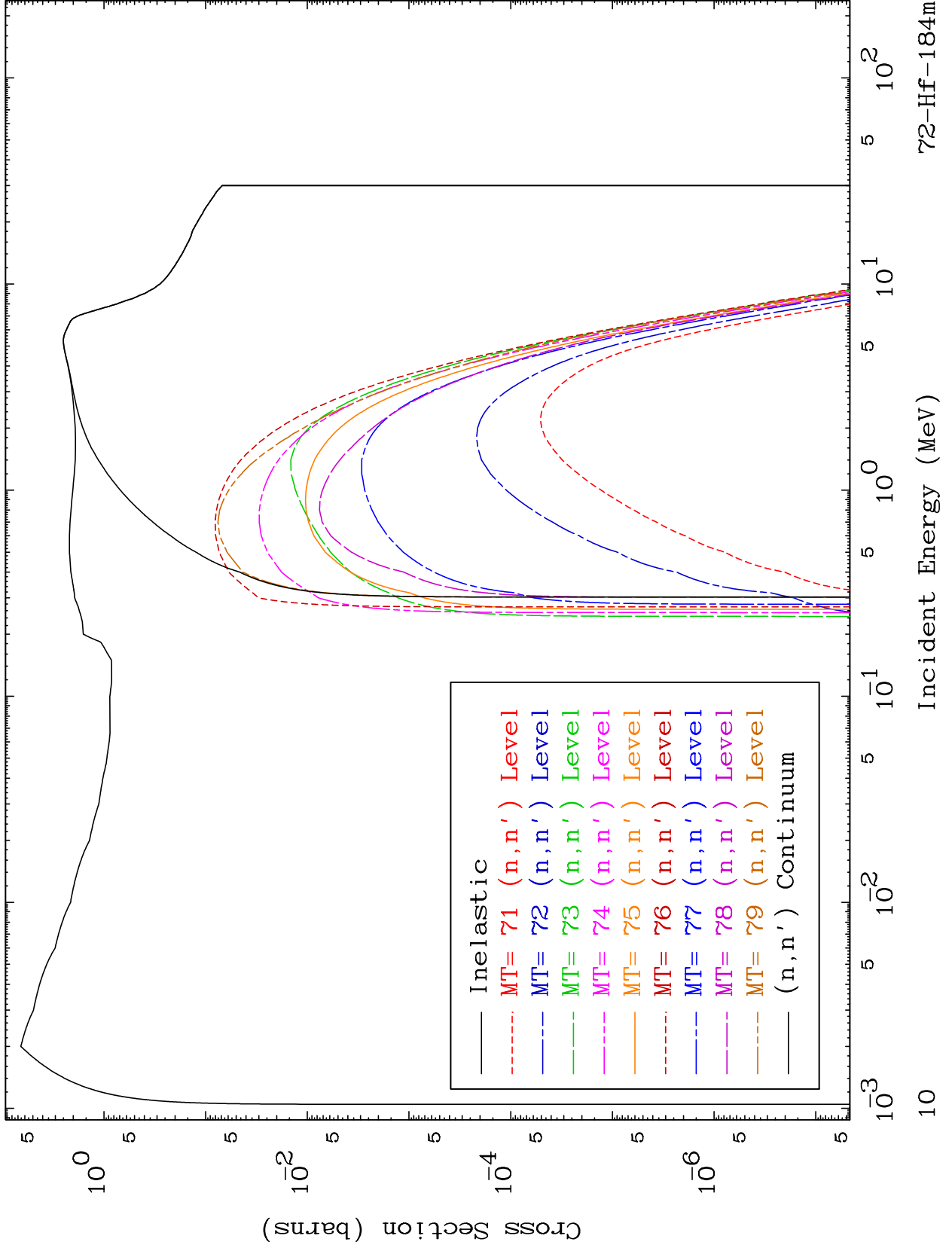
Incident Energy (MeV)

72-Hf-184m

MAT 7256

(n,n') Levels  
293 Kelvin Cross Sections

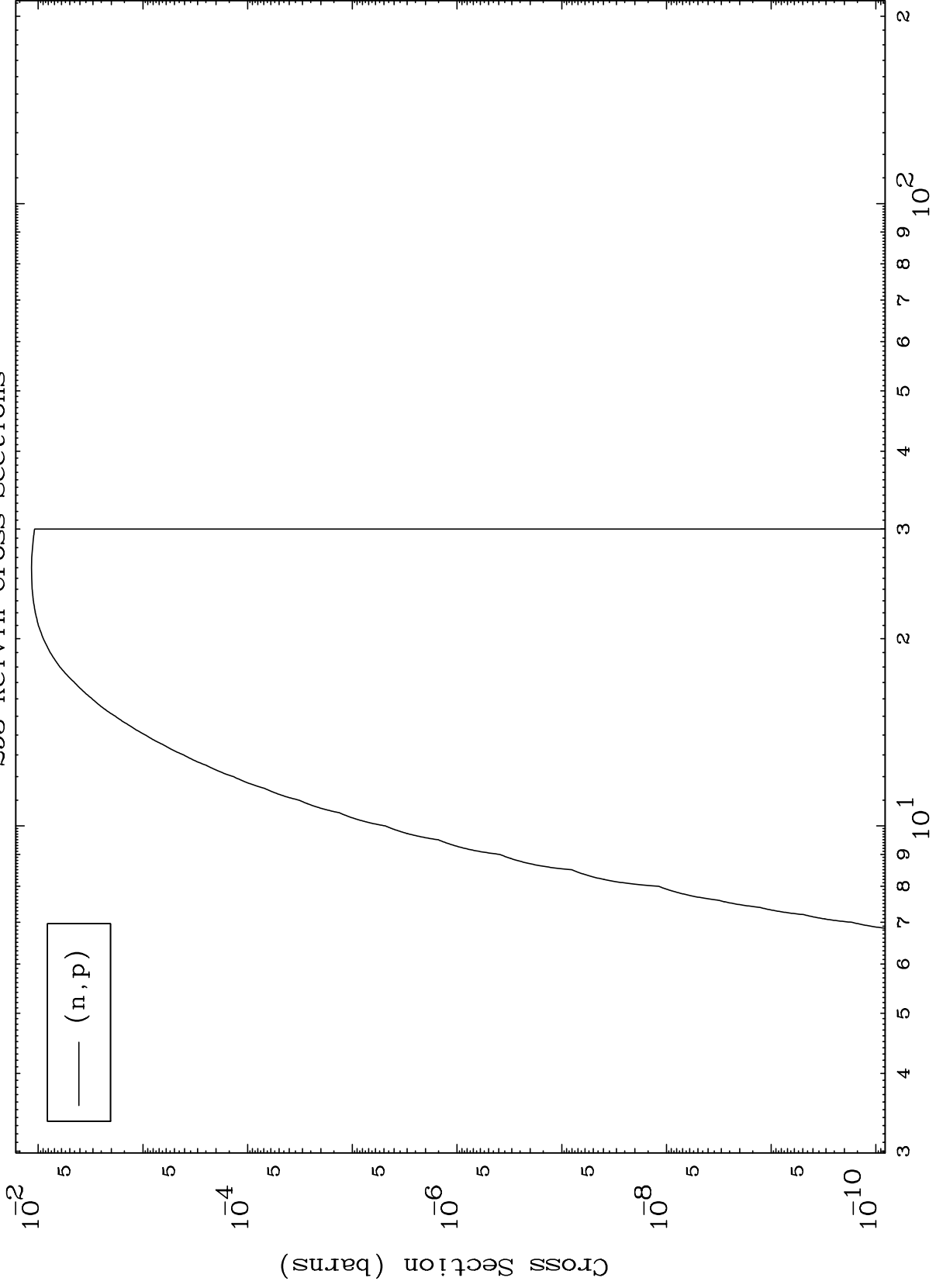
72-Hf-184m



MAT 7256

(n,p) Levels  
293 Kelvin Cross Sections

72-Hf-184m



(n,p)

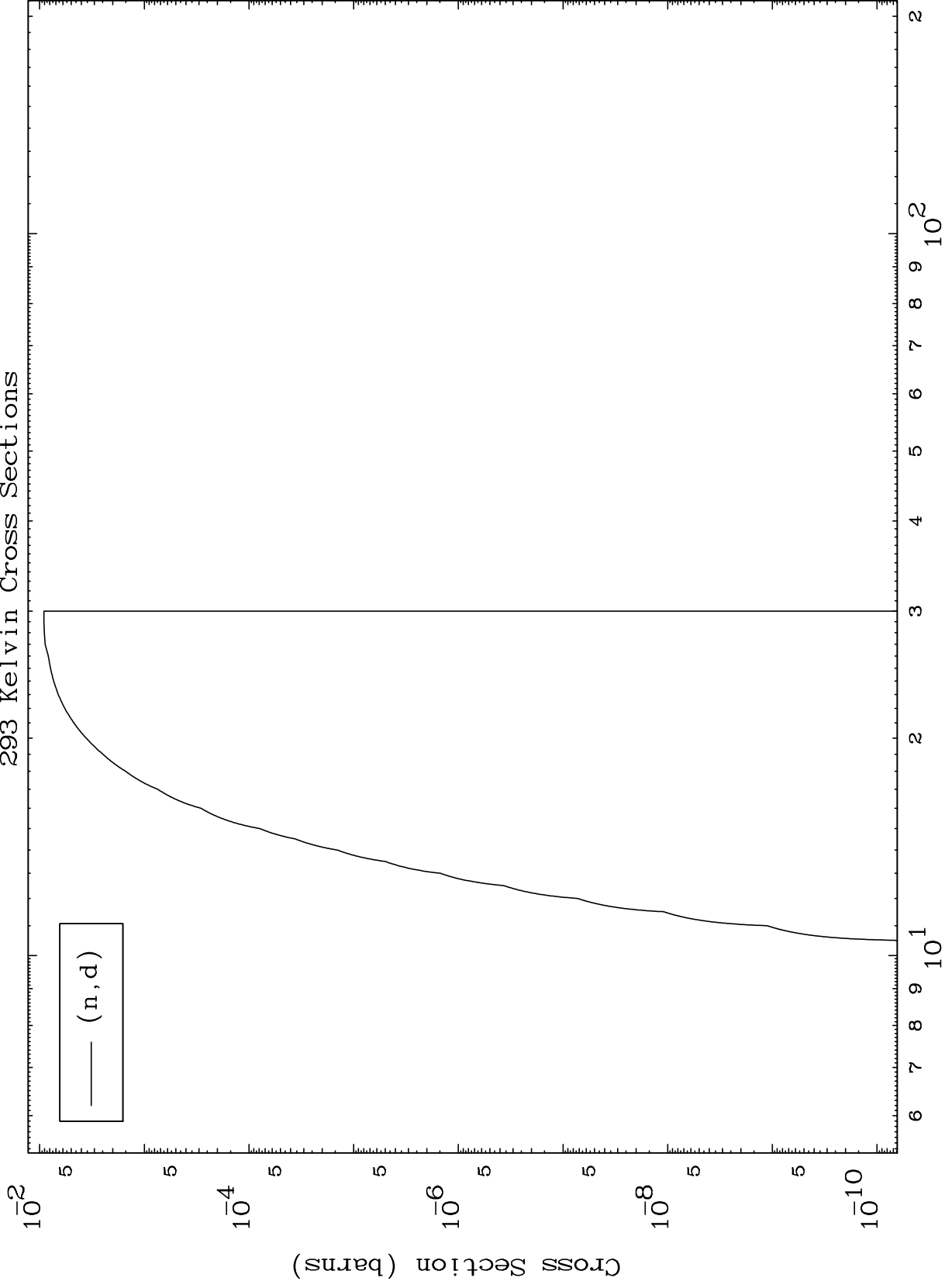
Incident Energy (MeV)

72-Hf-184m

MAT 7256

(n,d) Levels  
293 Kelvin Cross Sections

<sup>72</sup>Hf-184m



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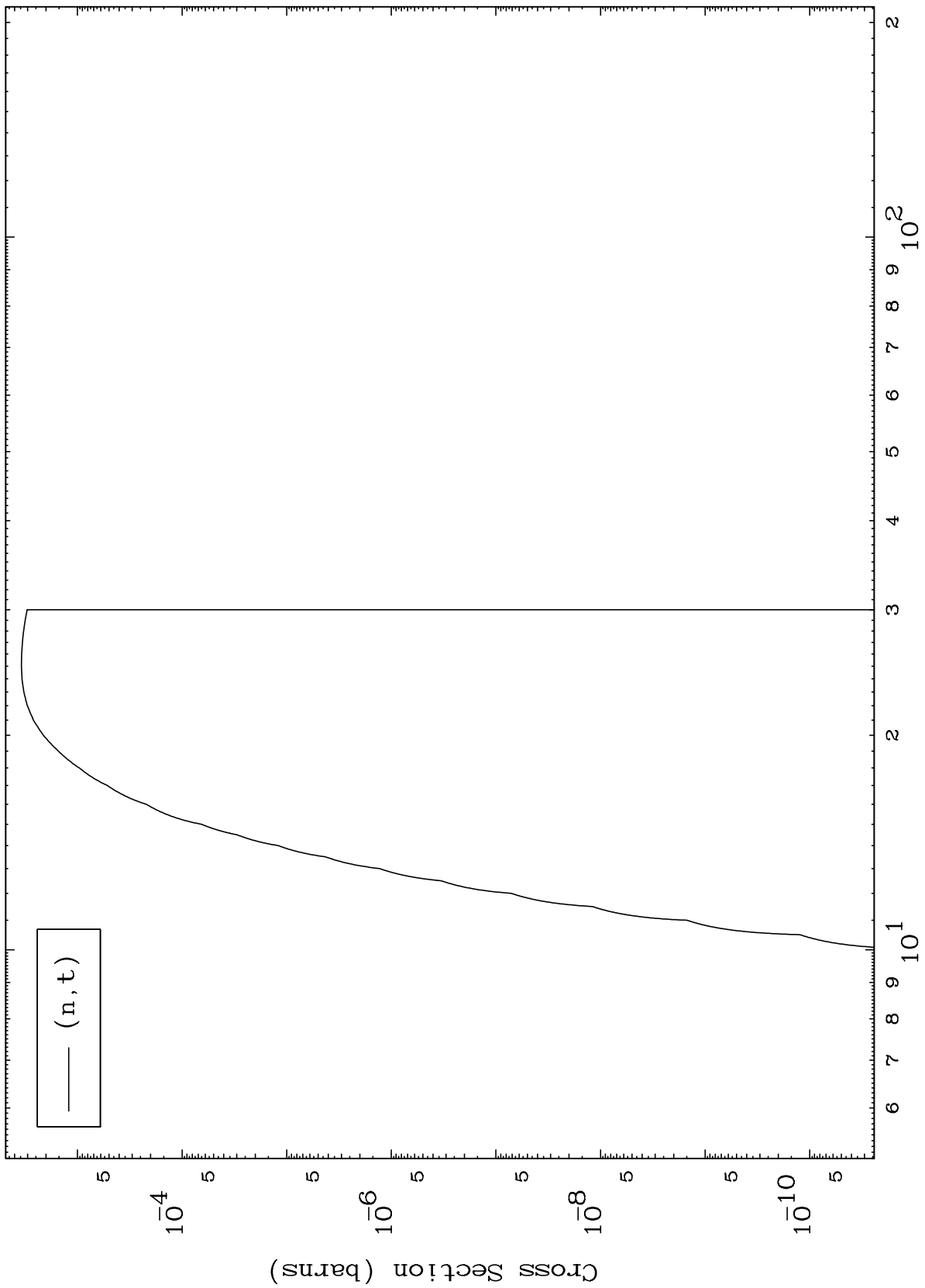
Incident Energy (MeV)

<sup>72</sup>Hf-184m

MAT 7256

(n, t) Levels  
293 Kelvin Cross Sections

72-Hf-184m



(n, t)

13

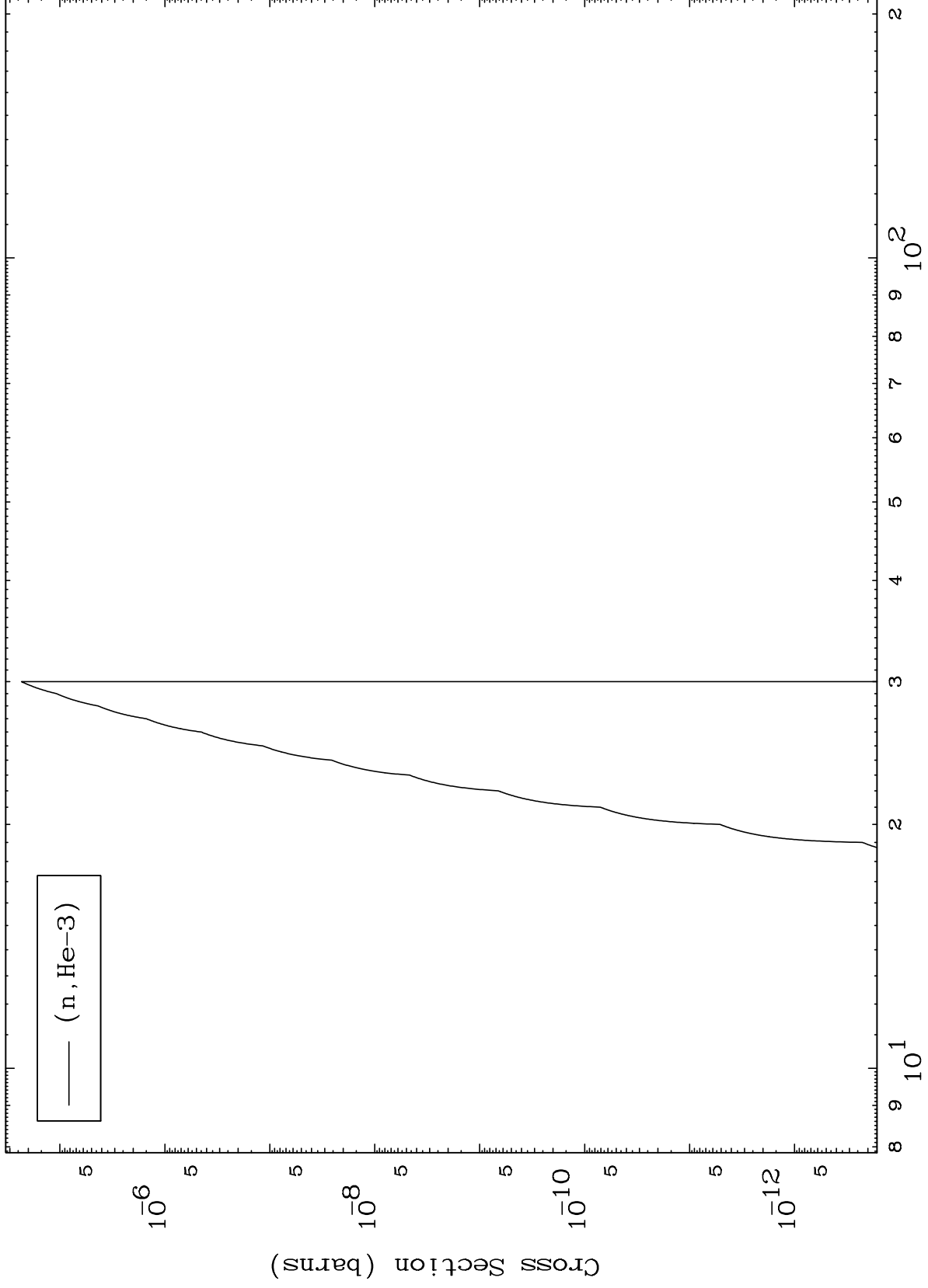
Incident Energy (MeV)

72-Hf-184m

MAT 7256

(n,He3) Levels  
293 Kelvin Cross Sections

72-Hf-184m



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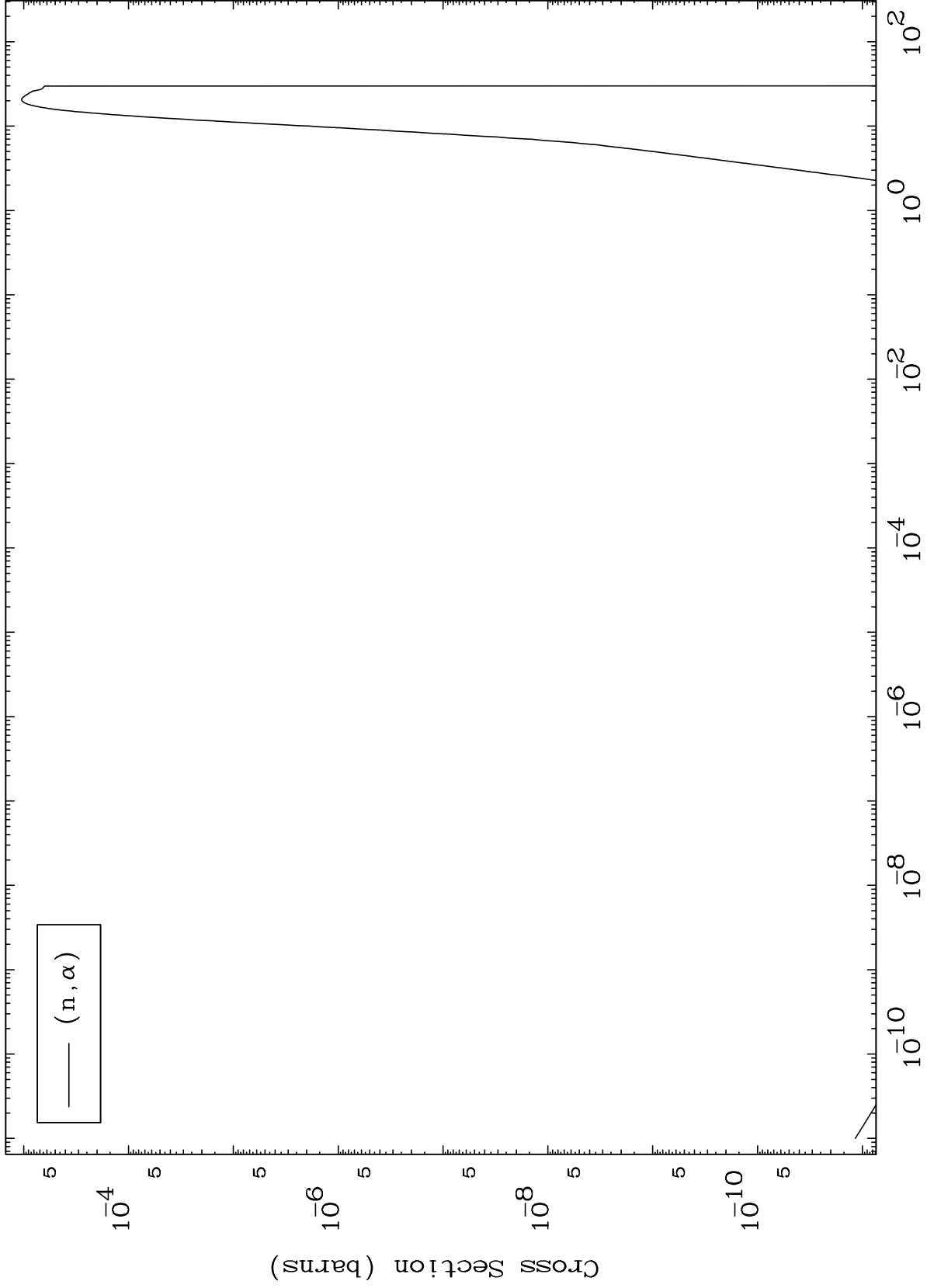
Incident Energy (MeV)

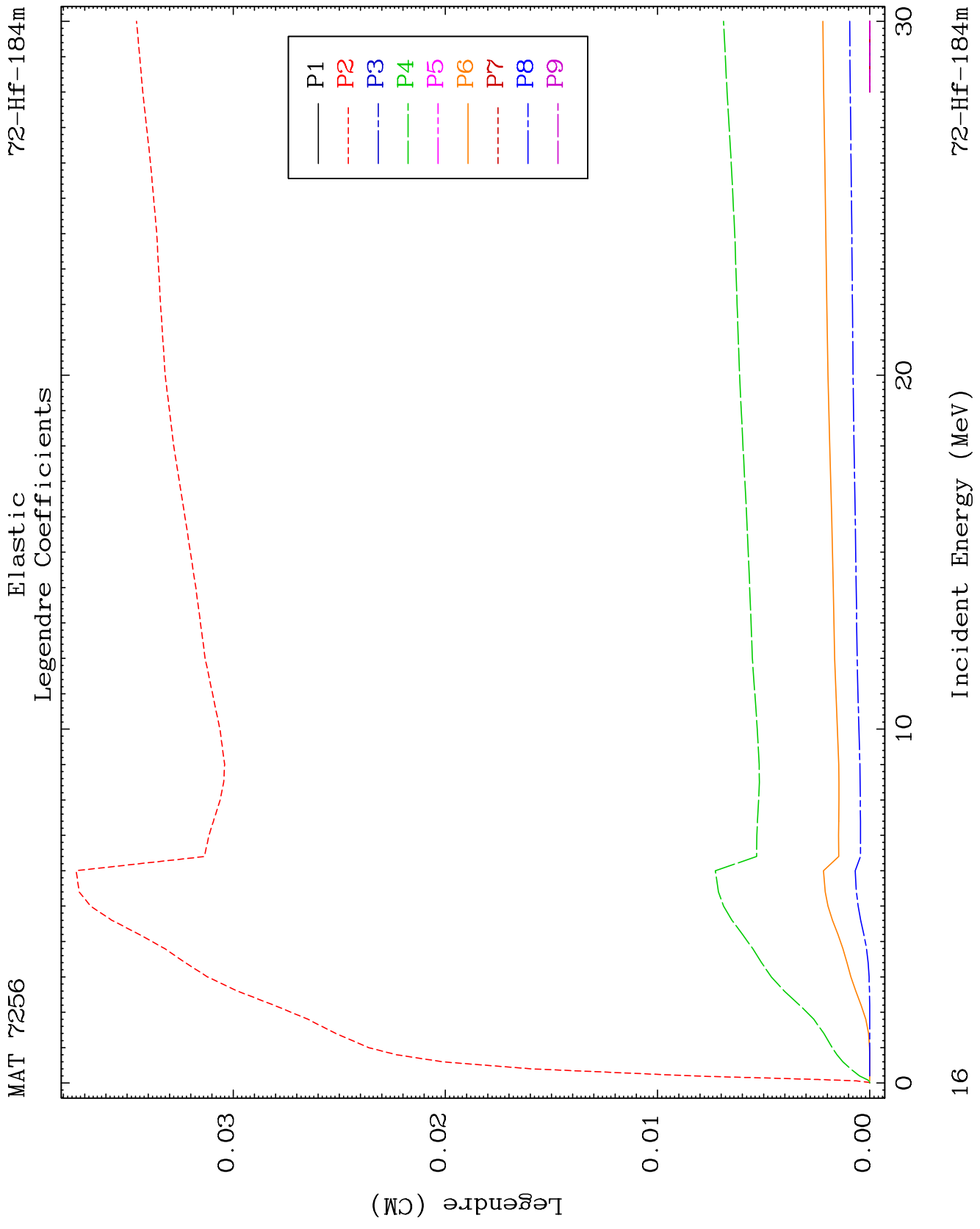
72-Hf-184m

MAT 7256

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

72-Hf-184m

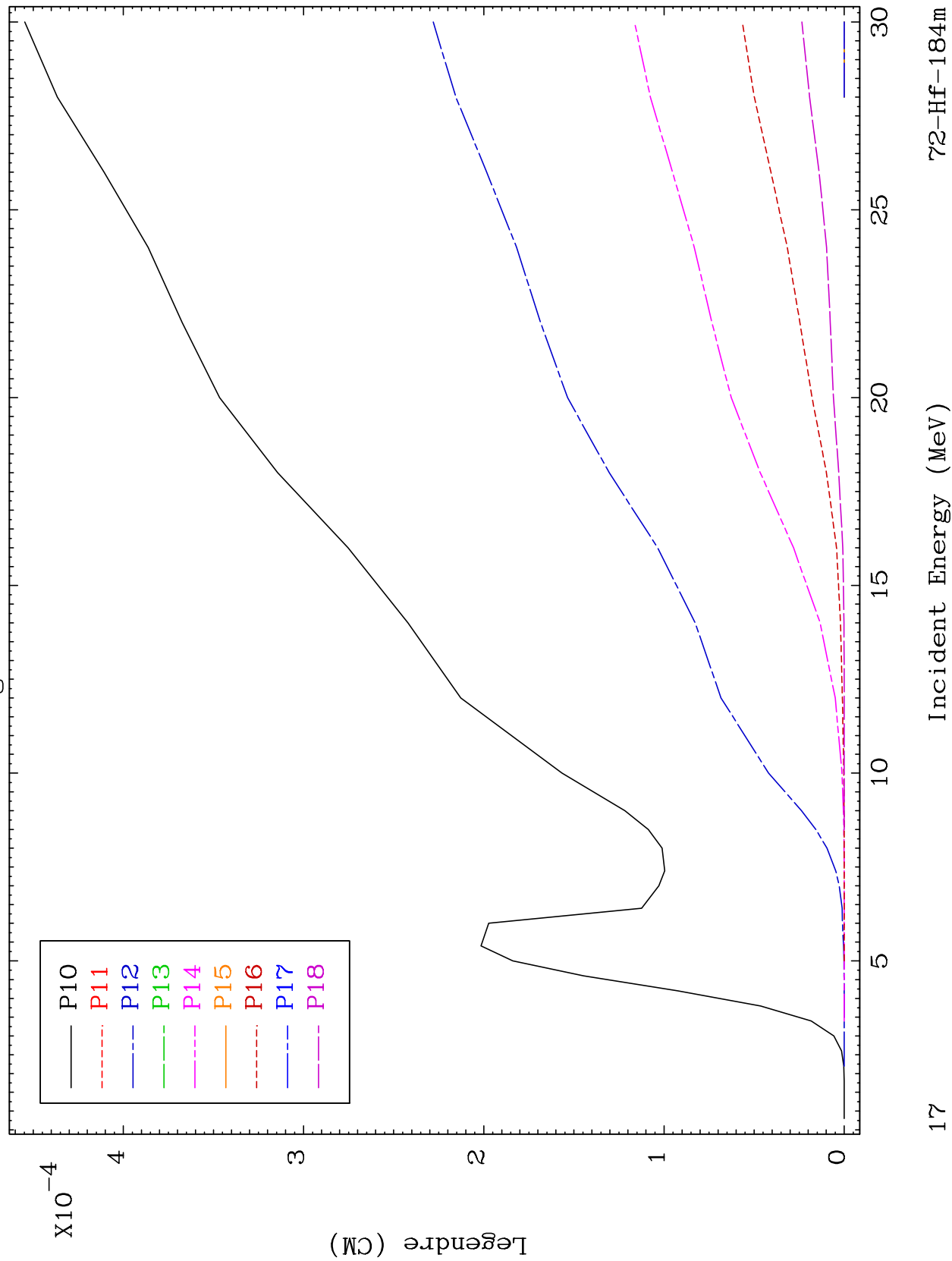




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Elastic Legendre Coefficients

<sup>72</sup>Hf-184m

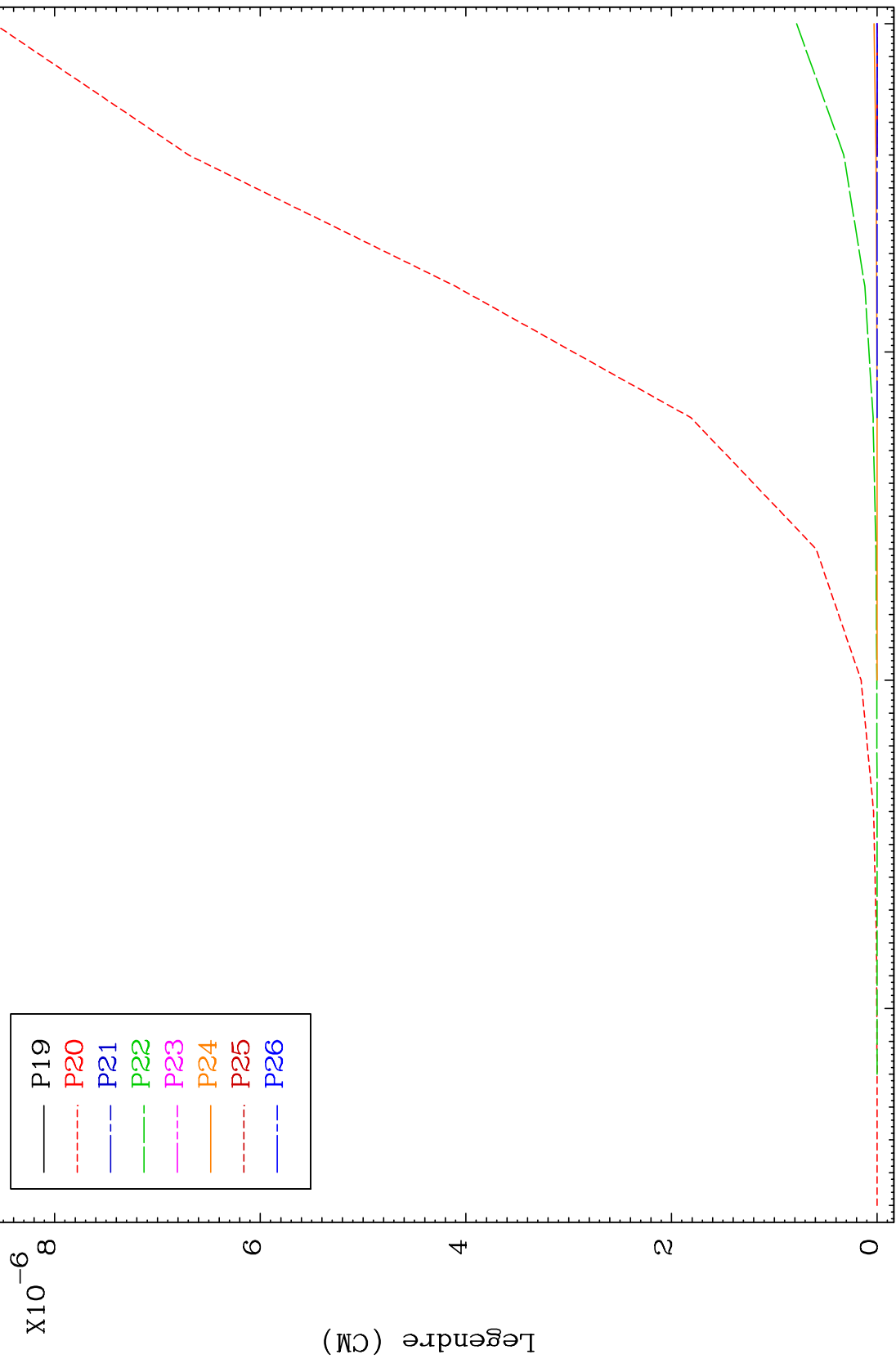
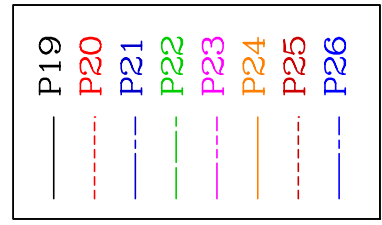


- P10
- P11
- P12
- P13
- P14
- P15
- P16
- P17
- P18

MAT 7256

Elastic  
Legendre Coefficients

72-Hf-184m



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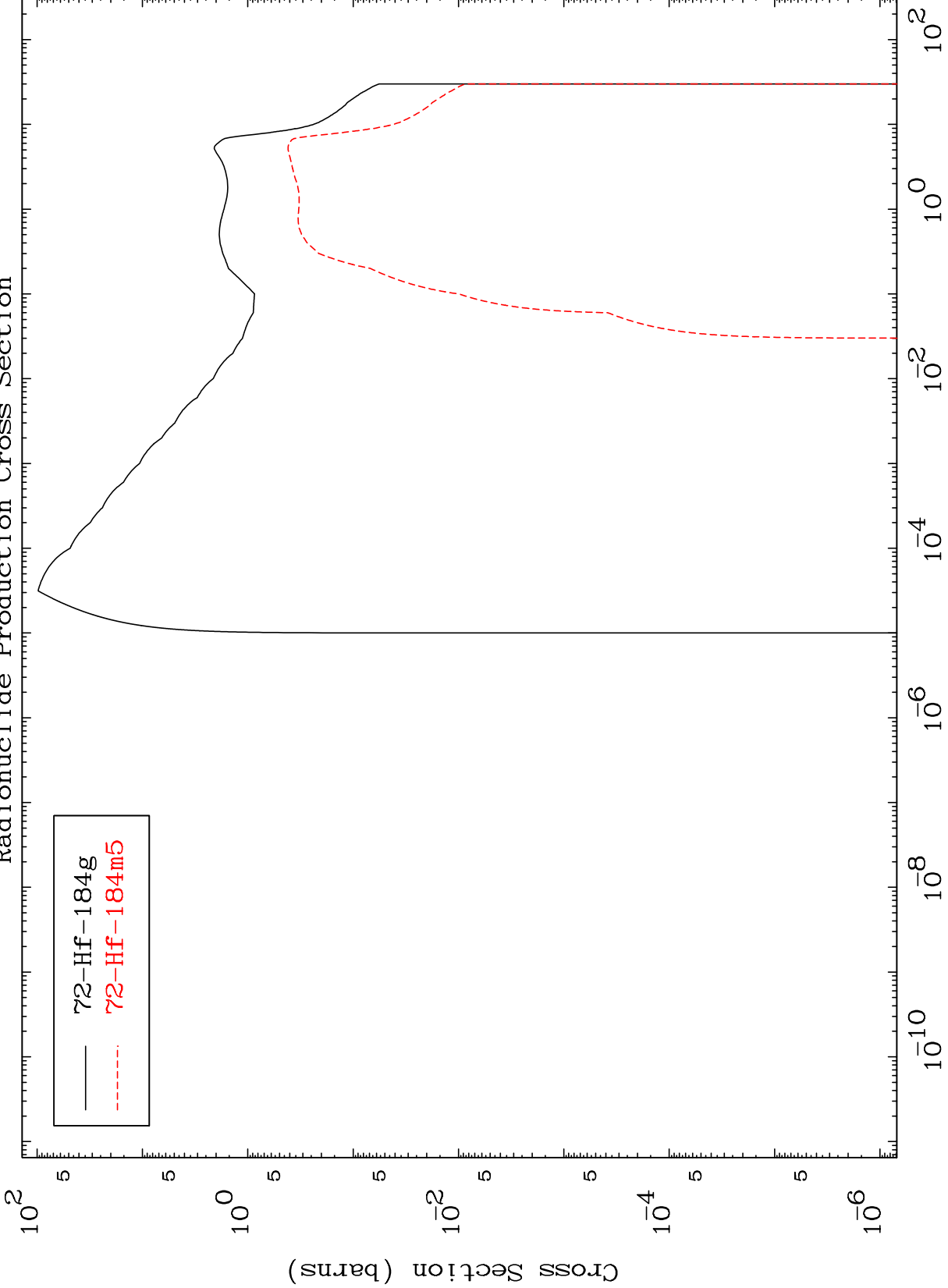
Incident Energy (MeV)

72-Hf-184m

MAT 7256

Radionuclide Production Cross Section

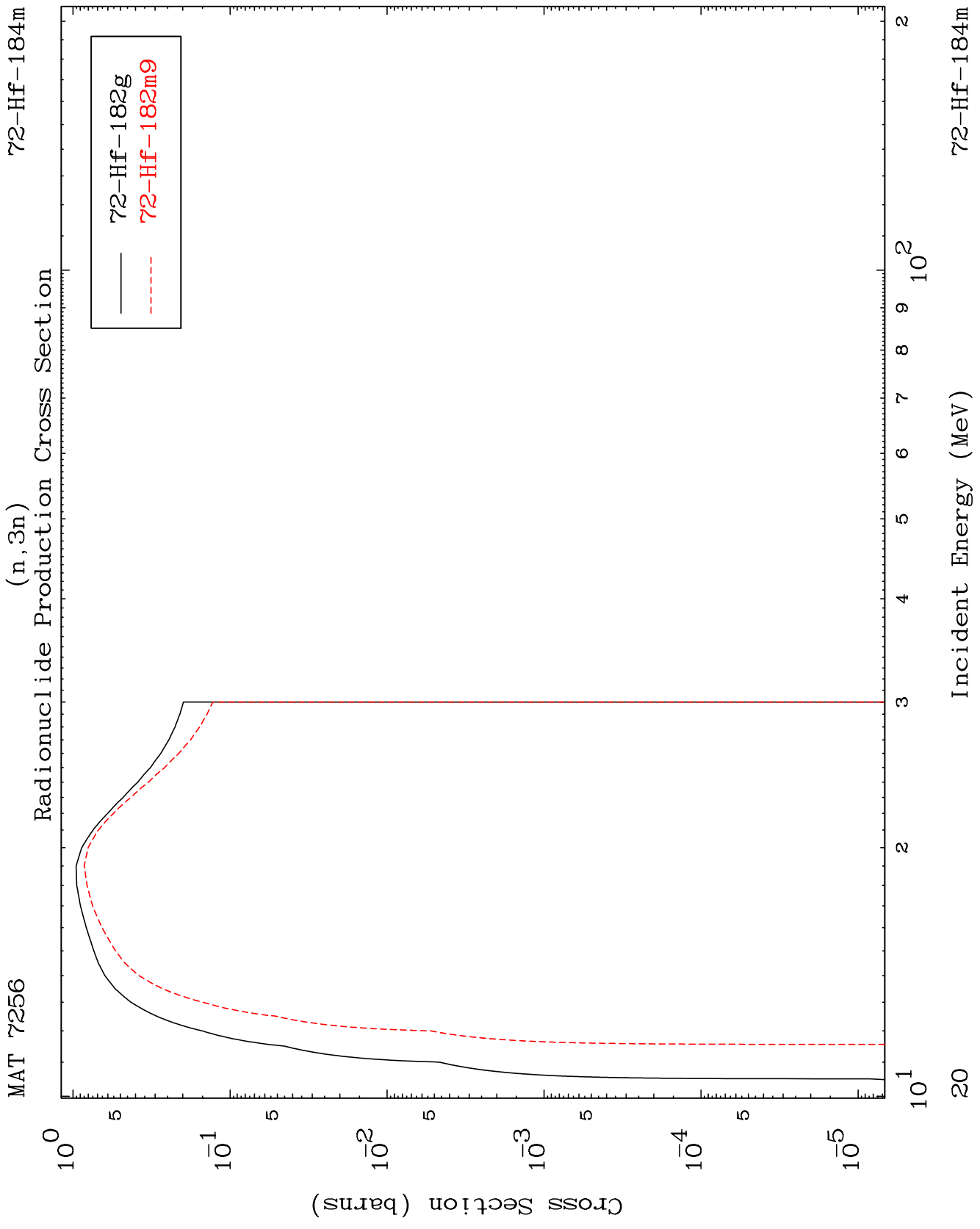
<sup>72</sup>Hf-184m



72-Hf-184g  
72-Hf-184m5

Incident Energy (MeV)

<sup>72</sup>Hf-184m



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Fission

<sup>72</sup>Hf-184m

Radionuclide Production Cross Section

