

Program EVALPLOT
(Version 2021-1)

by

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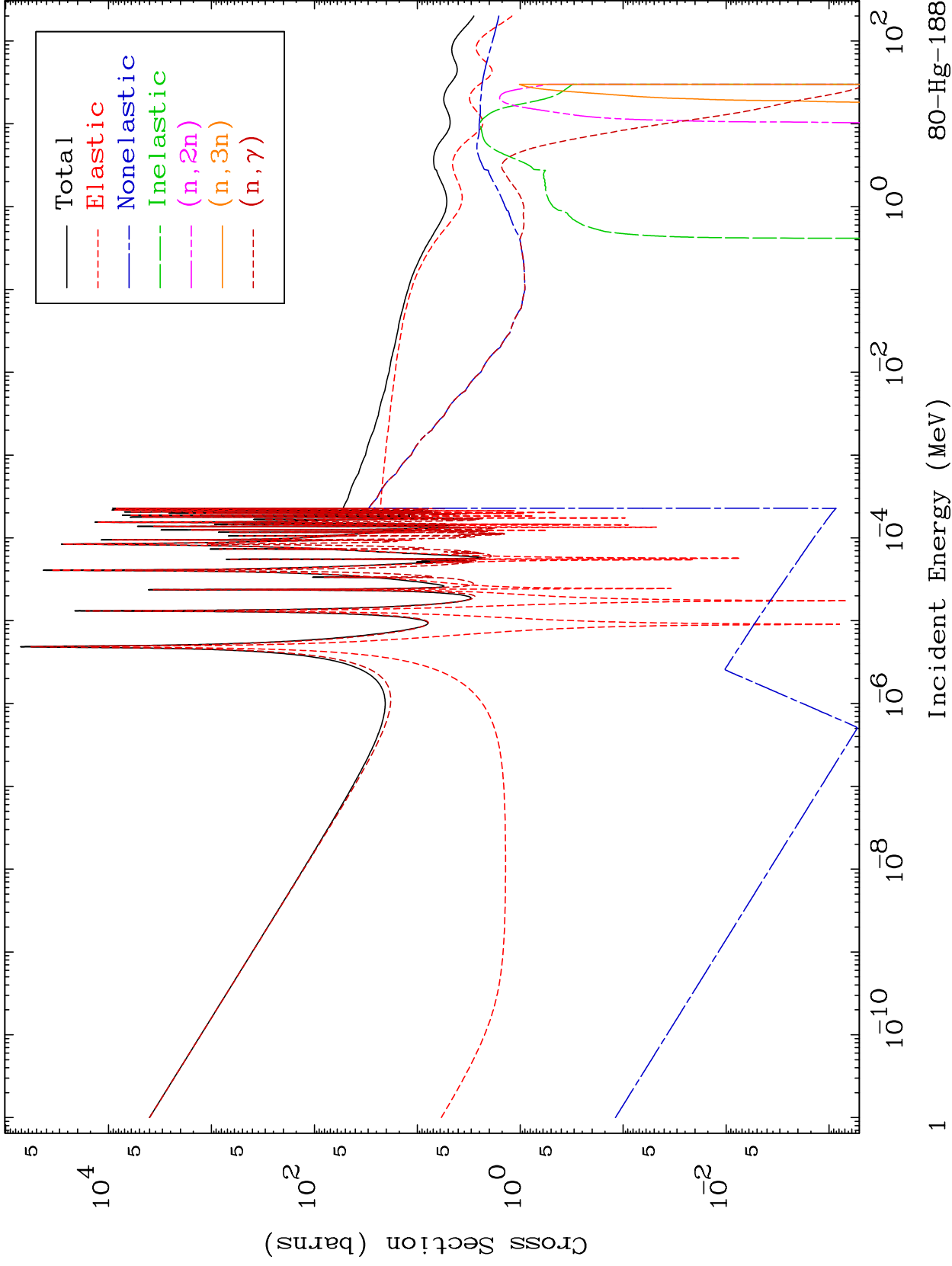
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8001

Neutron Major
293 Kelvin Cross Sections

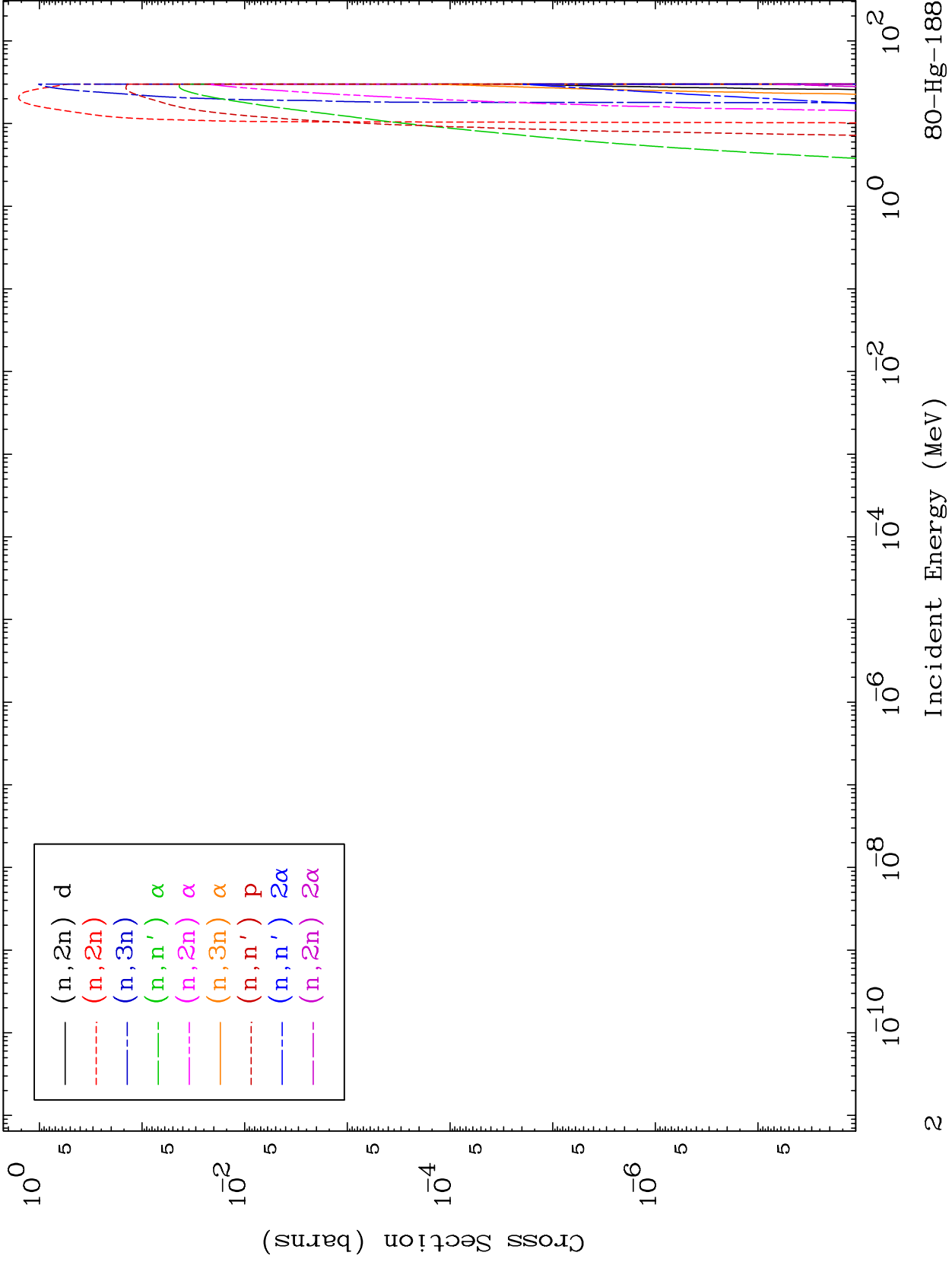
80-Hg-188



MAT 8001

Neutron Absorption
293 Kelvin Cross Sections

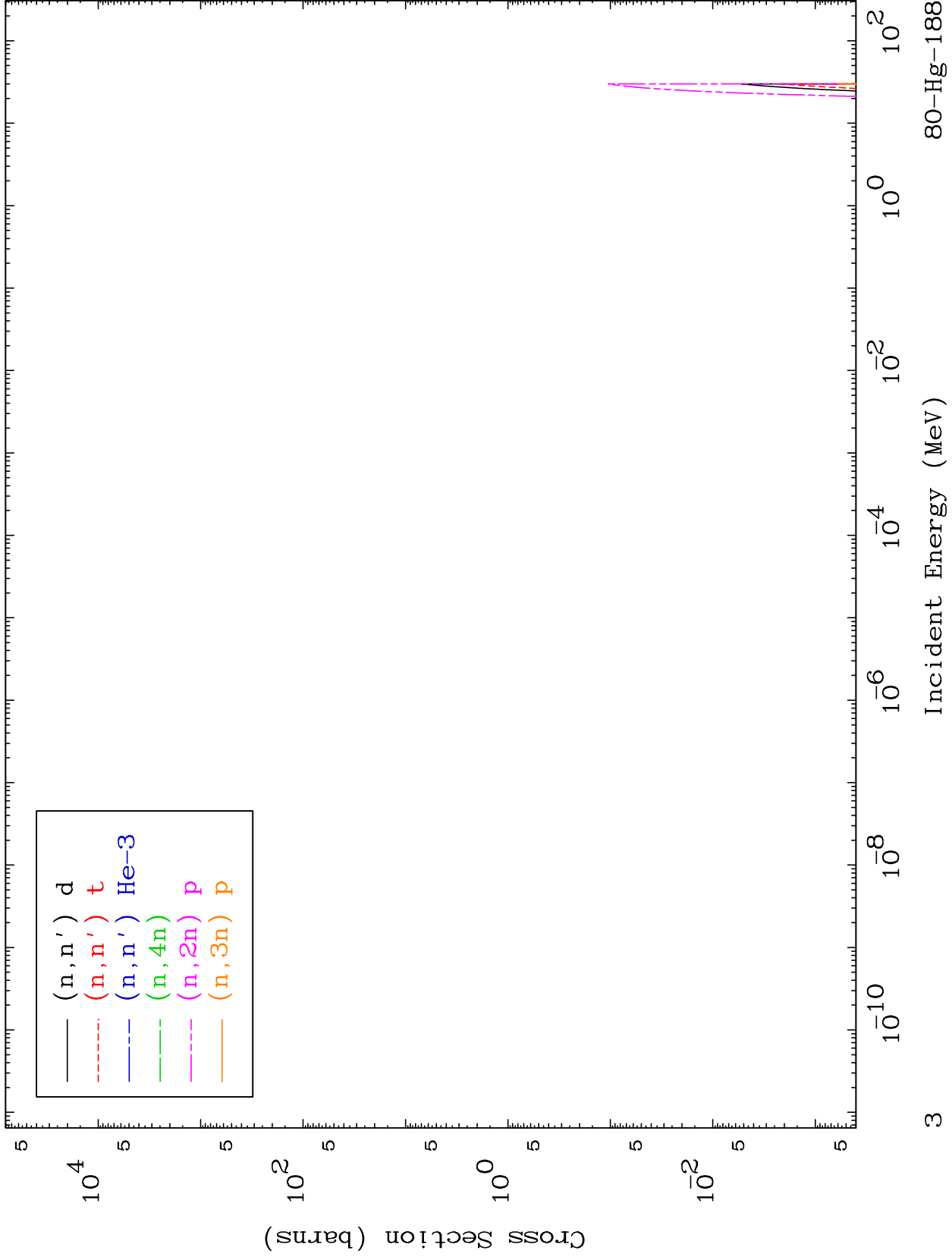
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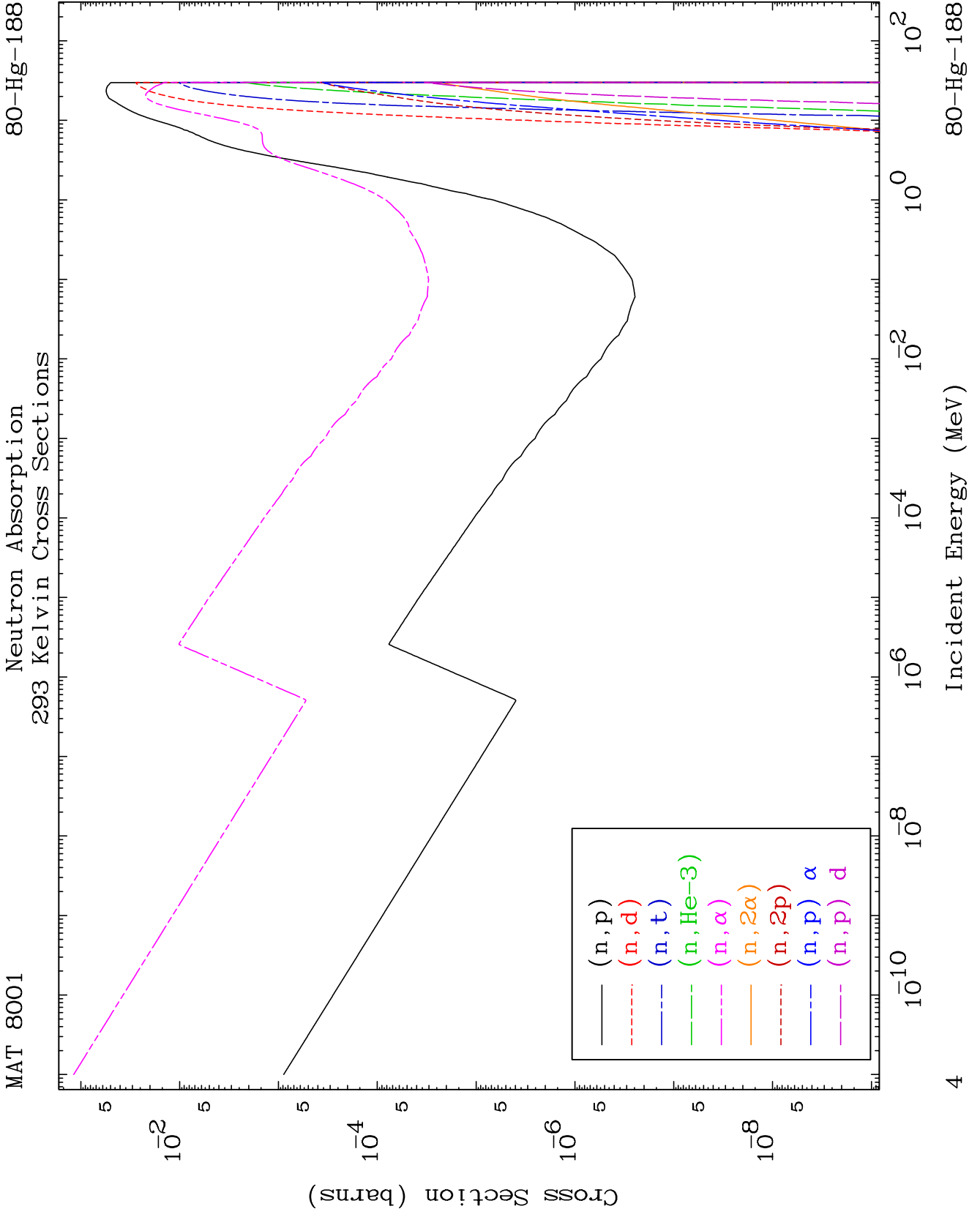


MAT 8001

Neutron Absorption
293 Kelvin Cross Sections

80-Hg-188

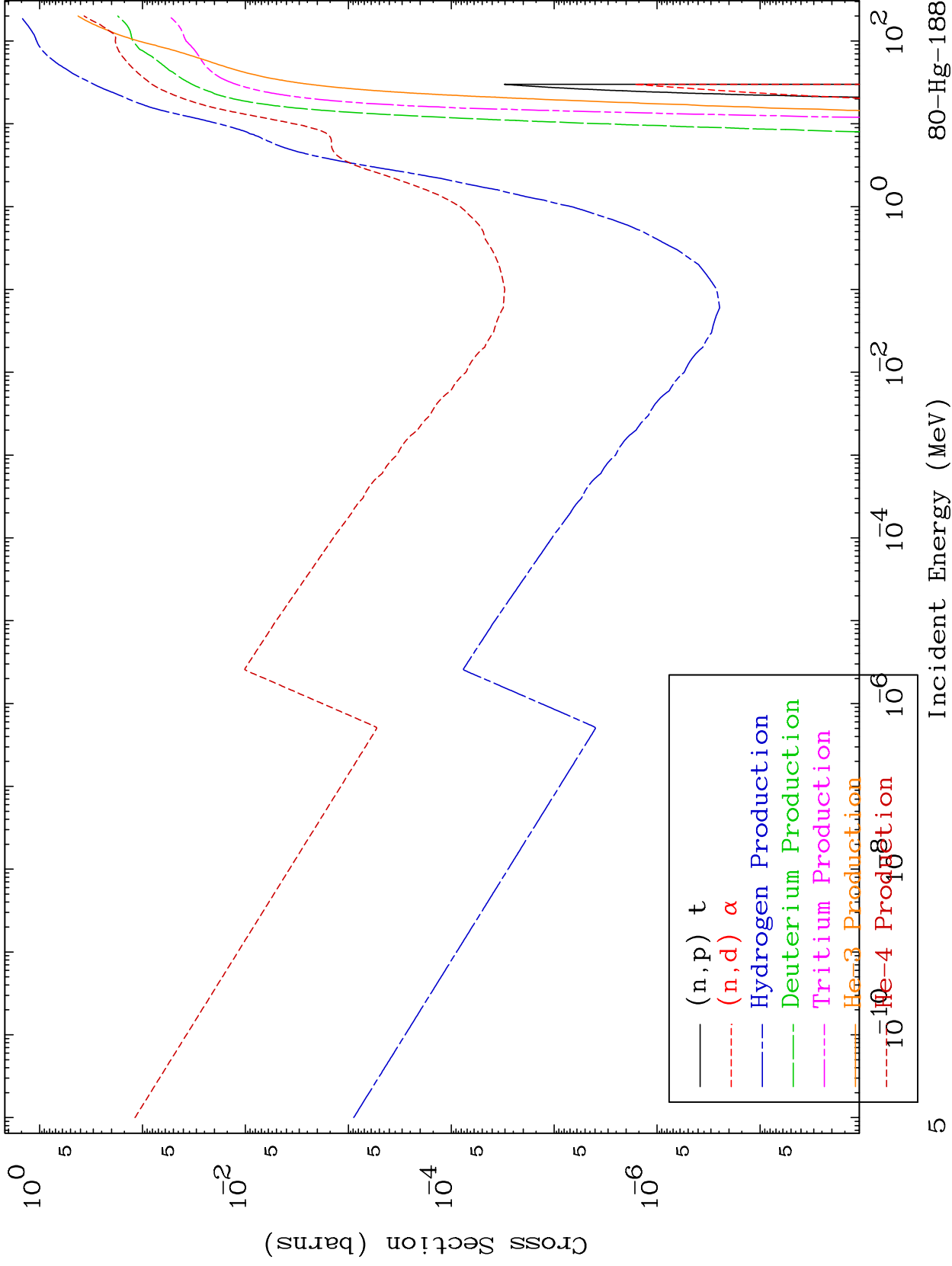




MAT 8001

Neutron Absorption
293 Kelvin Cross Sections

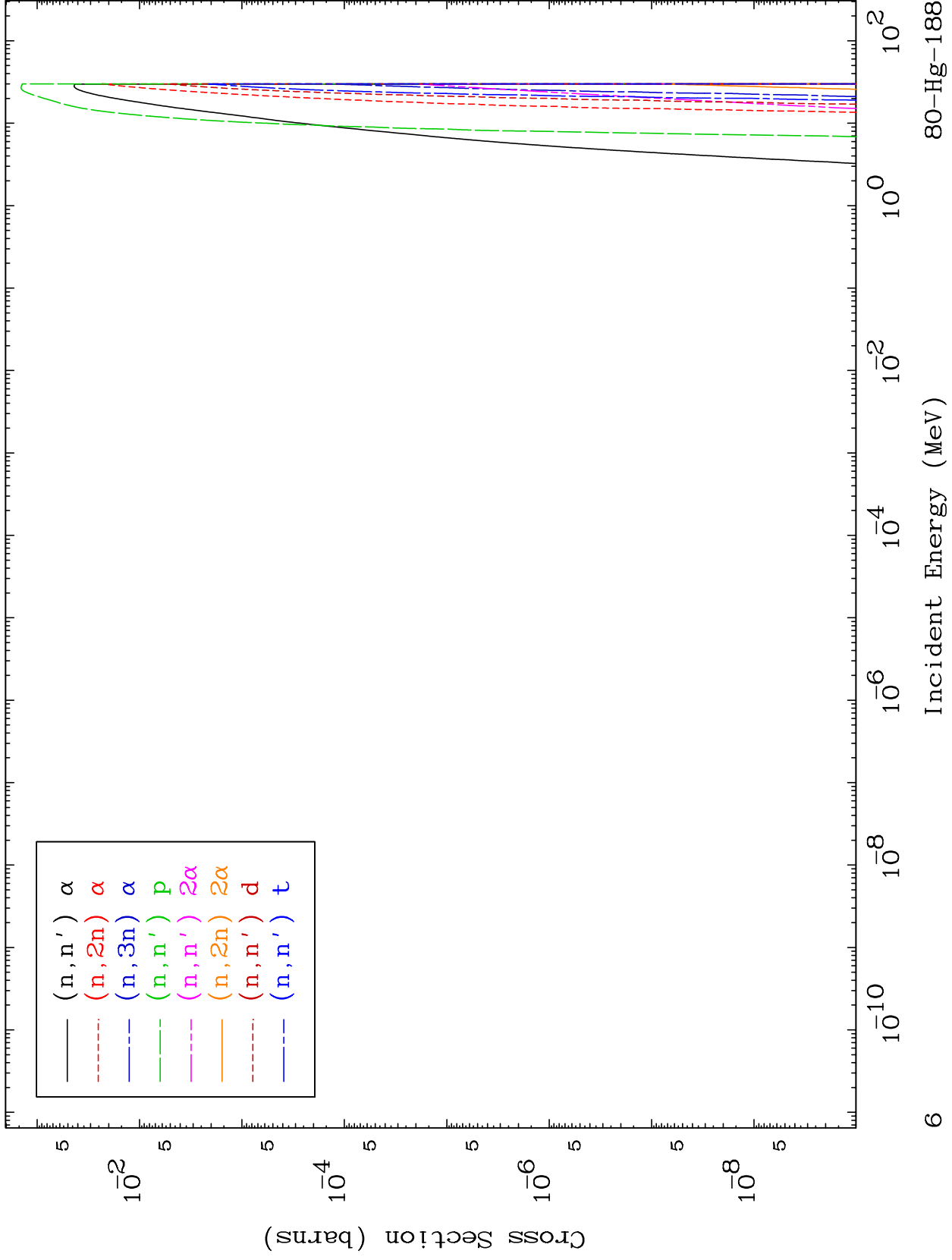
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MAT 8001

Charged Particle
293 Kelvin Cross Sections

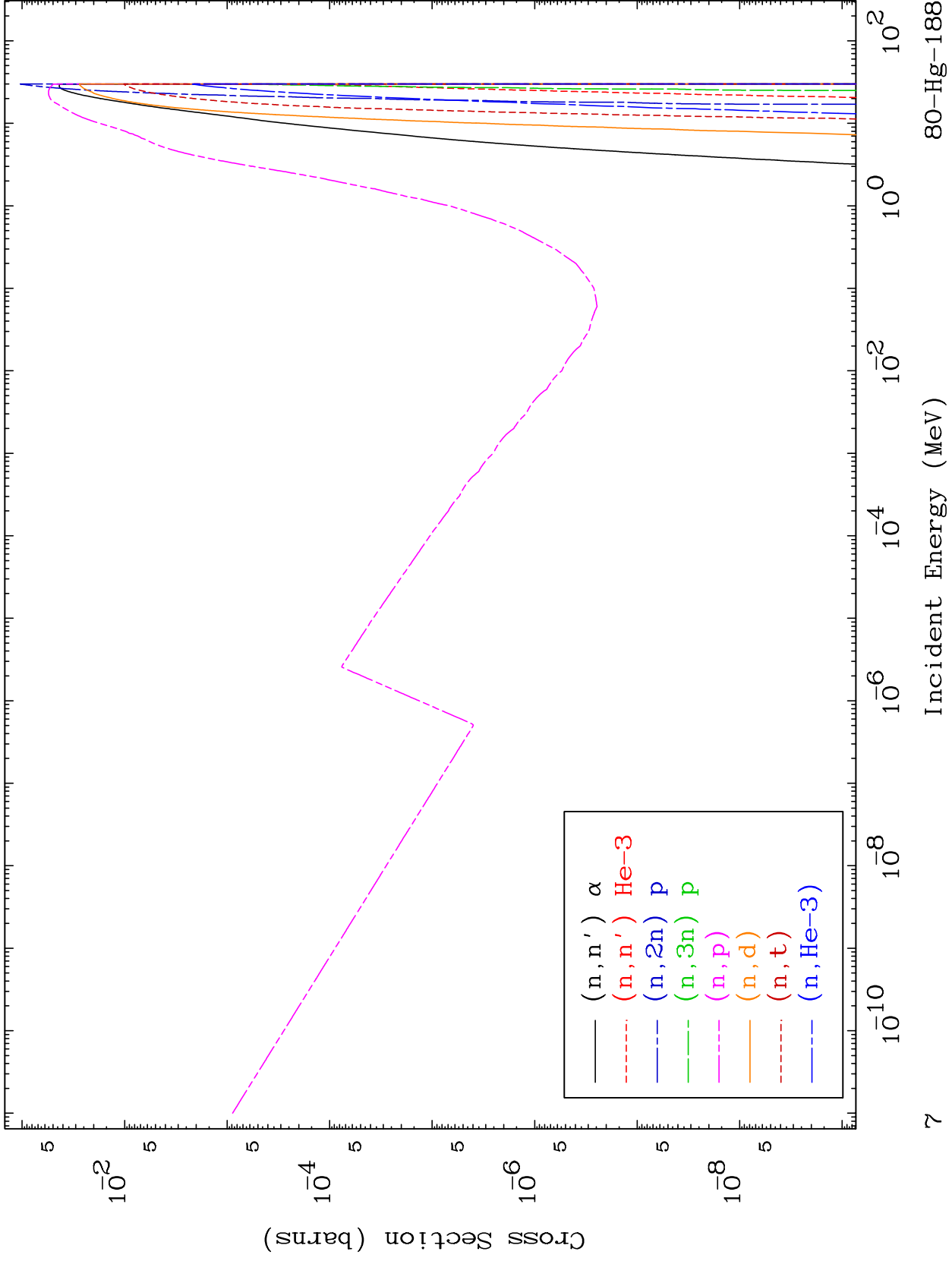
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MAT 8001

Charged Particle
293 Kelvin Cross Sections

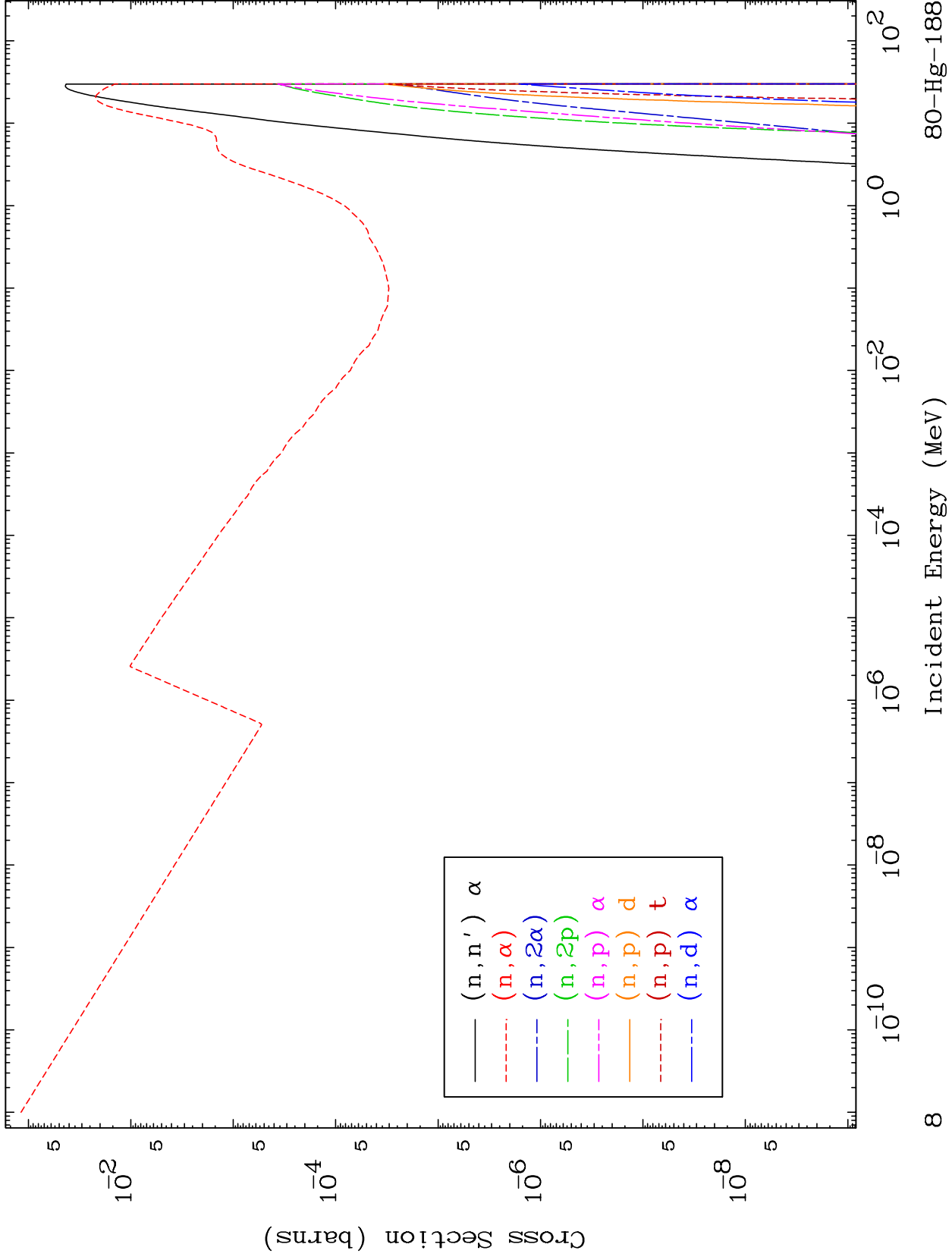
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MAT 8001

Charged Particle
293 Kelvin Cross Sections

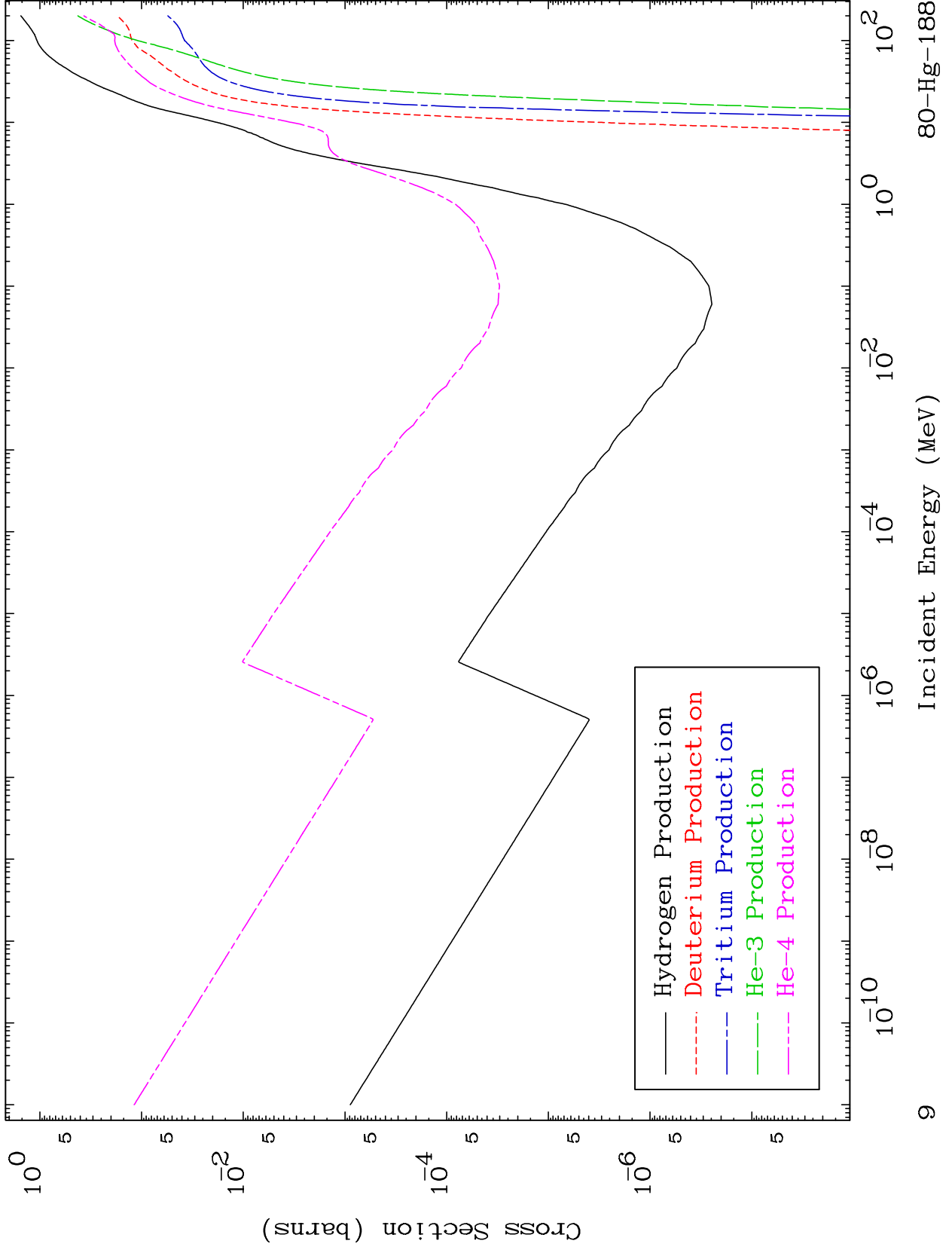
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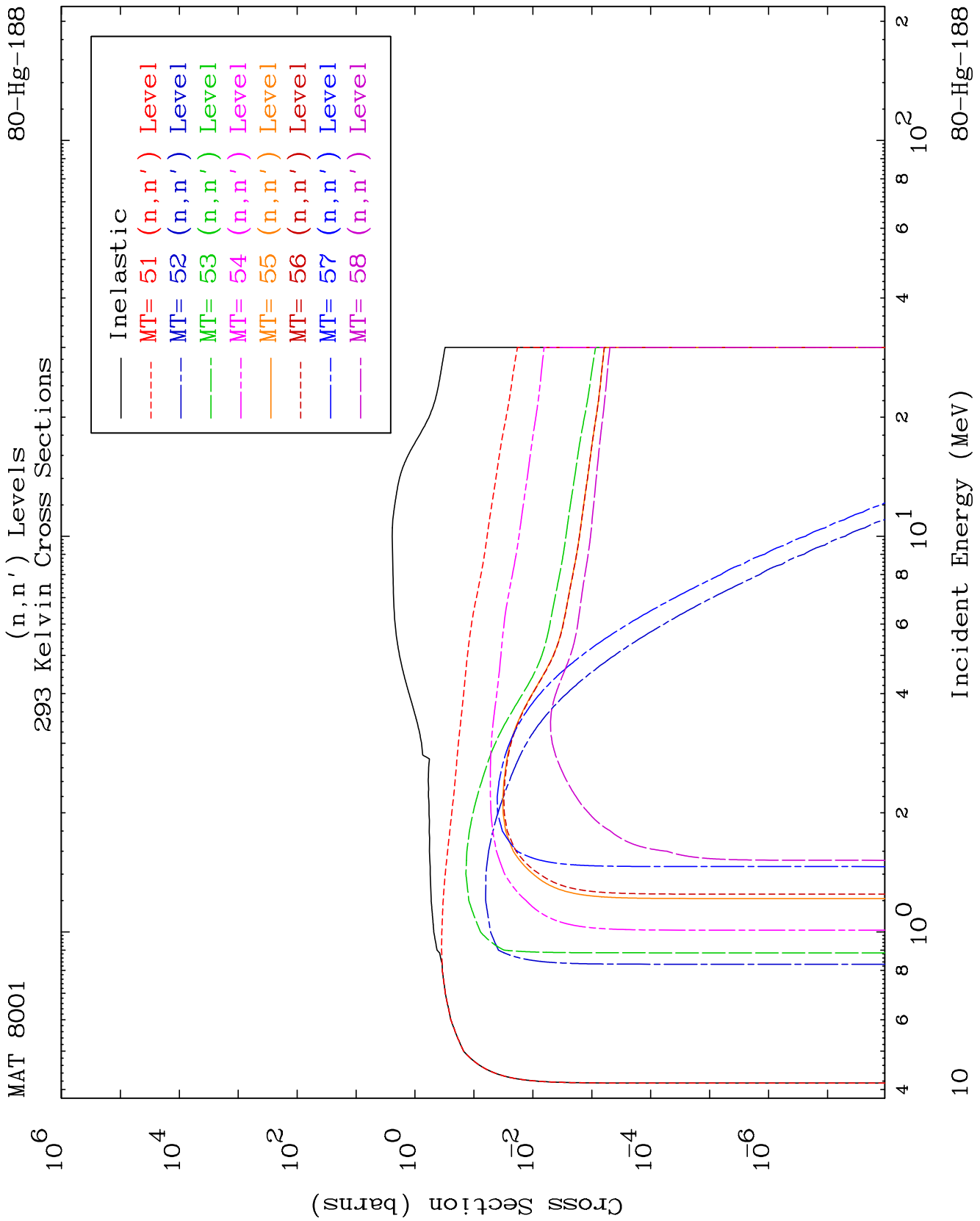


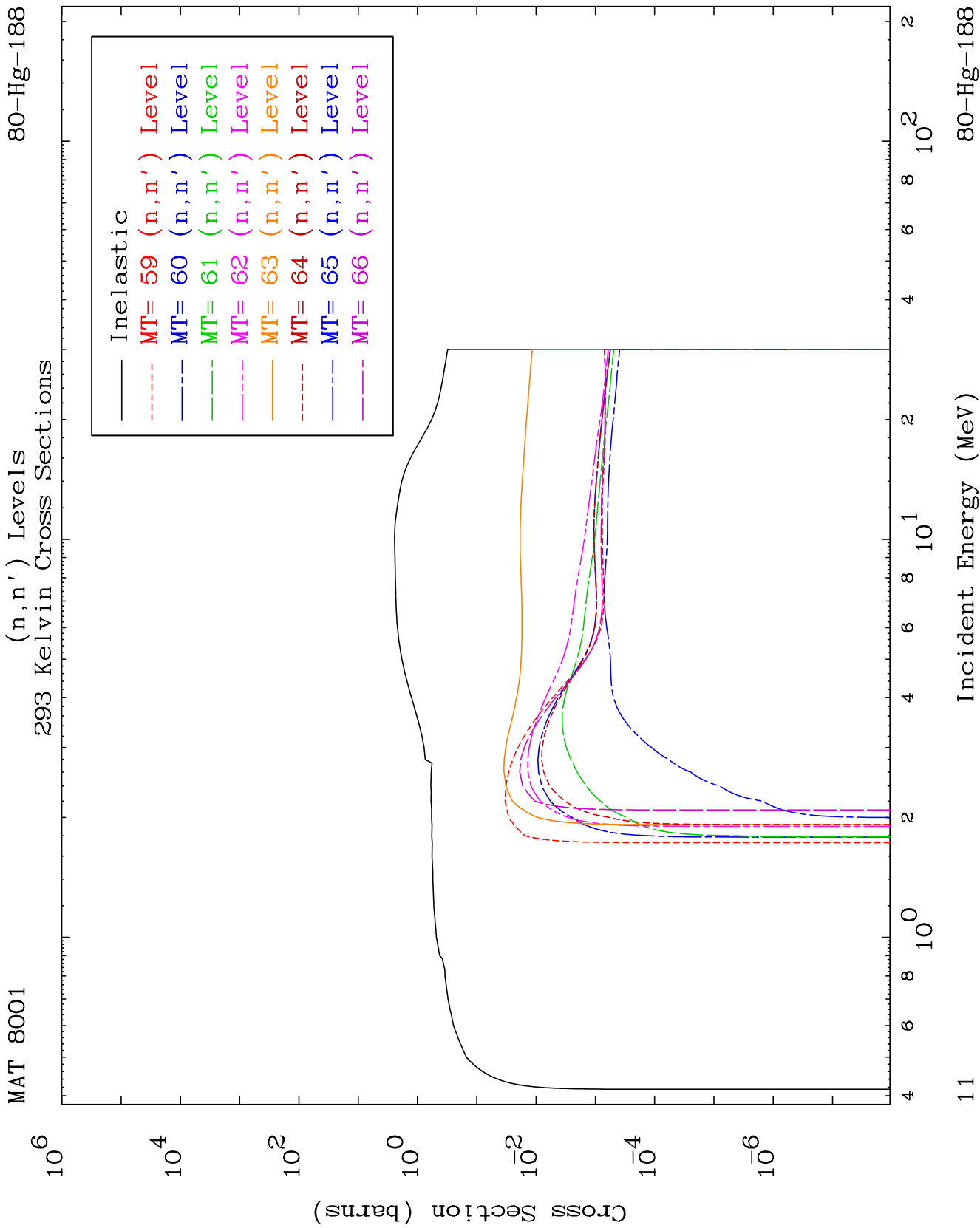
MAT 8001

Particle Production
293 Kelvin Cross Sections

80-Hg-188



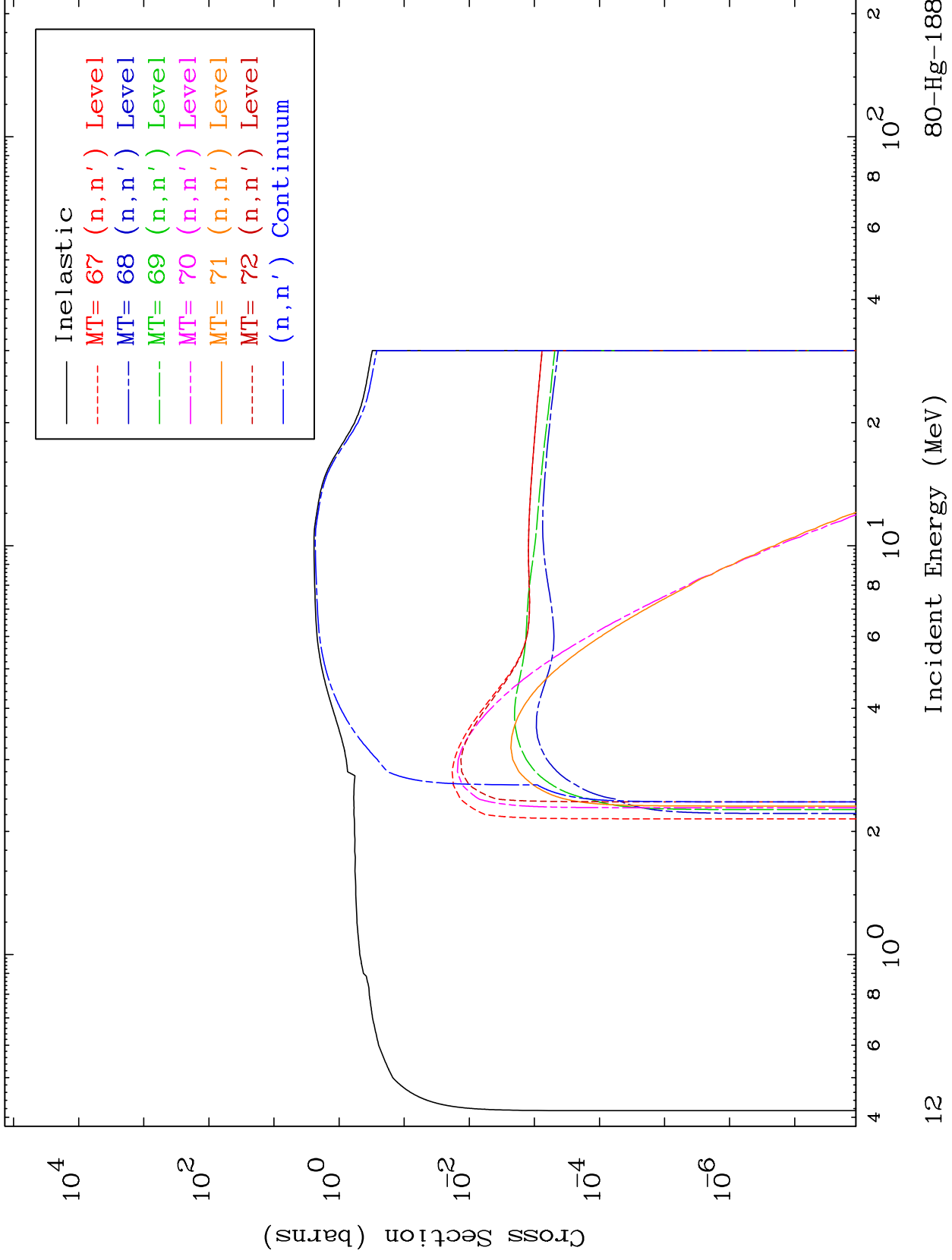




MAT 8001

(n,n') Levels
293 Kelvin Cross Sections

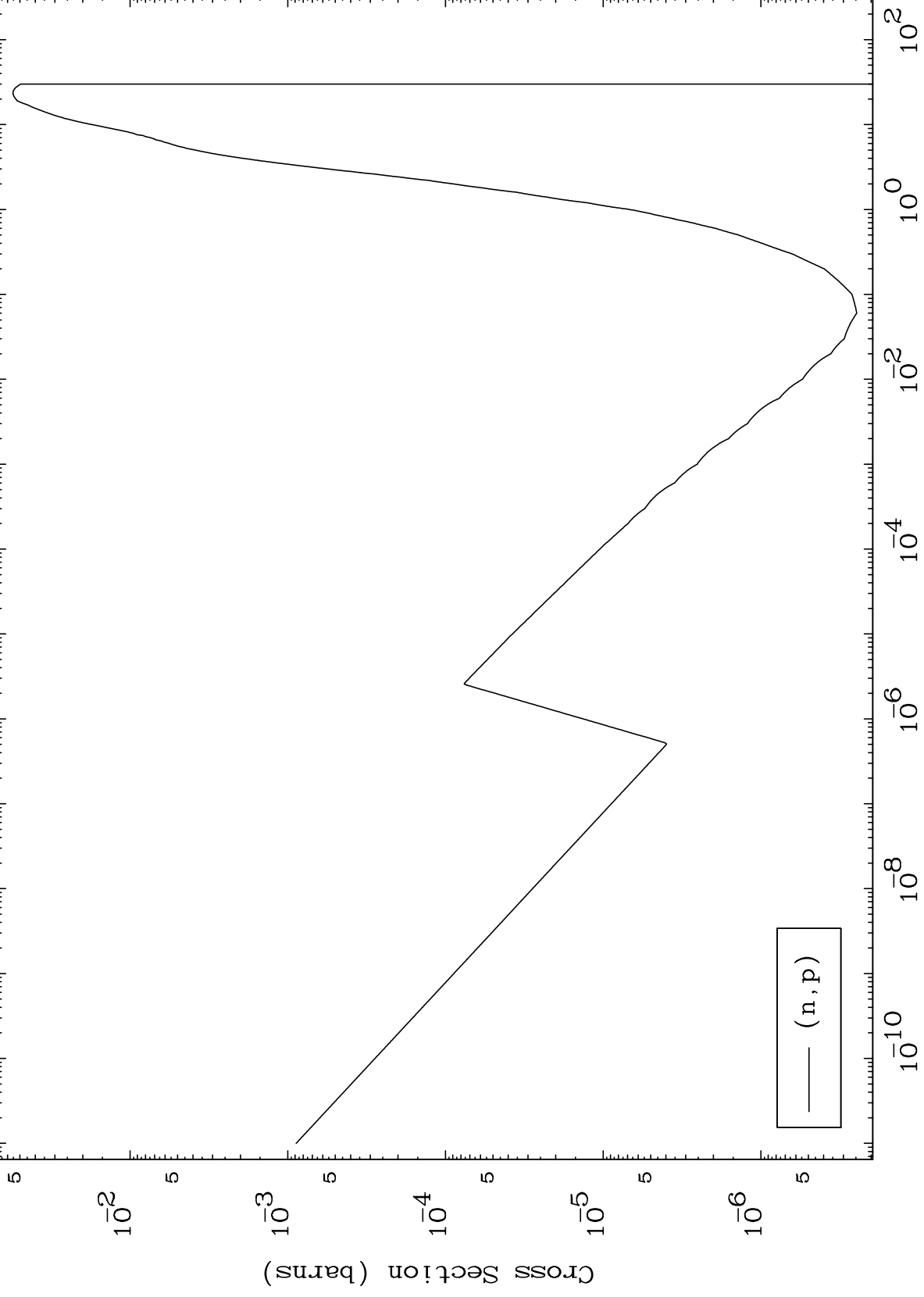
80-Hg-188



MAT 8001

(n,p) Levels
293 Kelvin Cross Sections

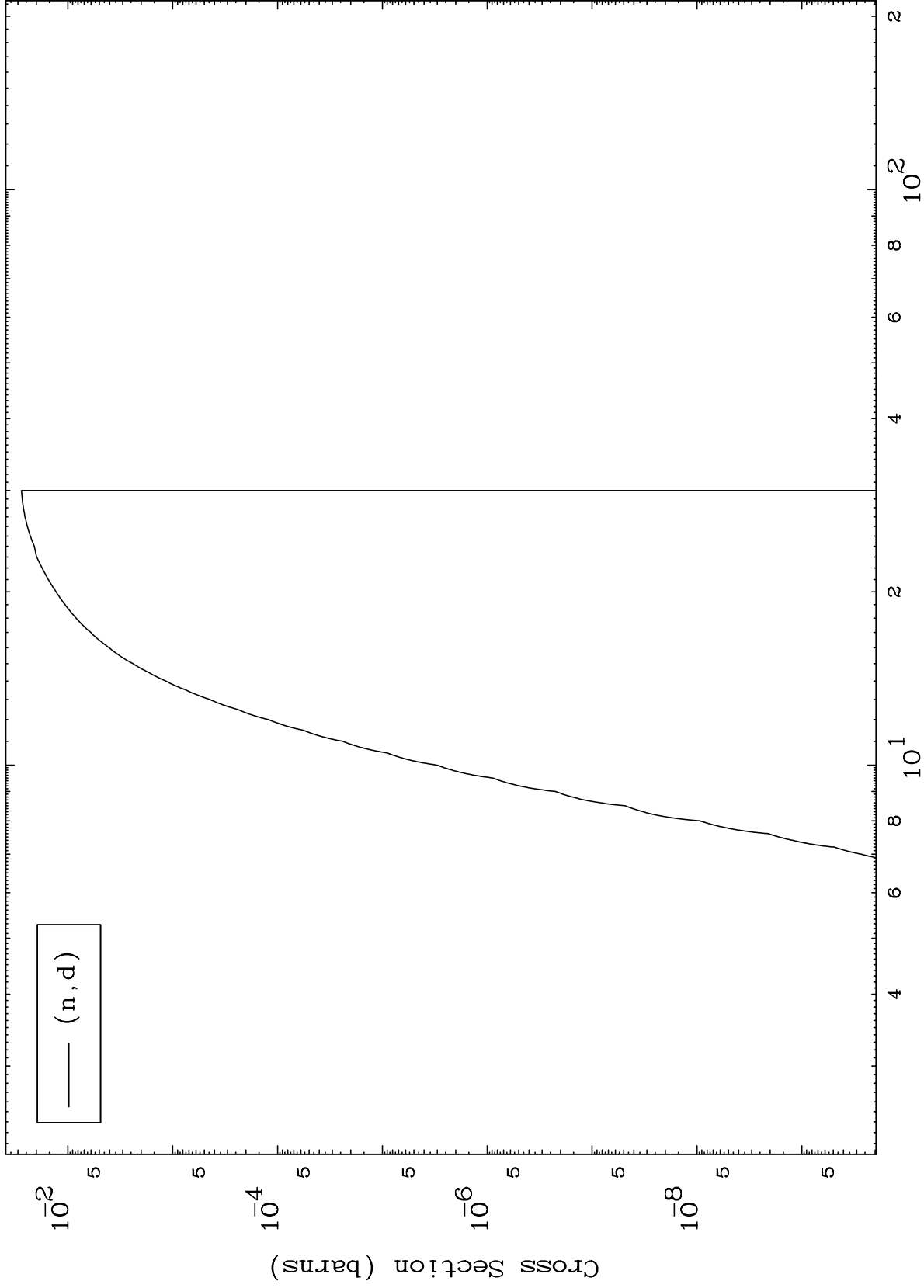
80-Hg-188



MAT 8001

(n,d) Levels
293 Kelvin Cross Sections

80-Hg-188



14

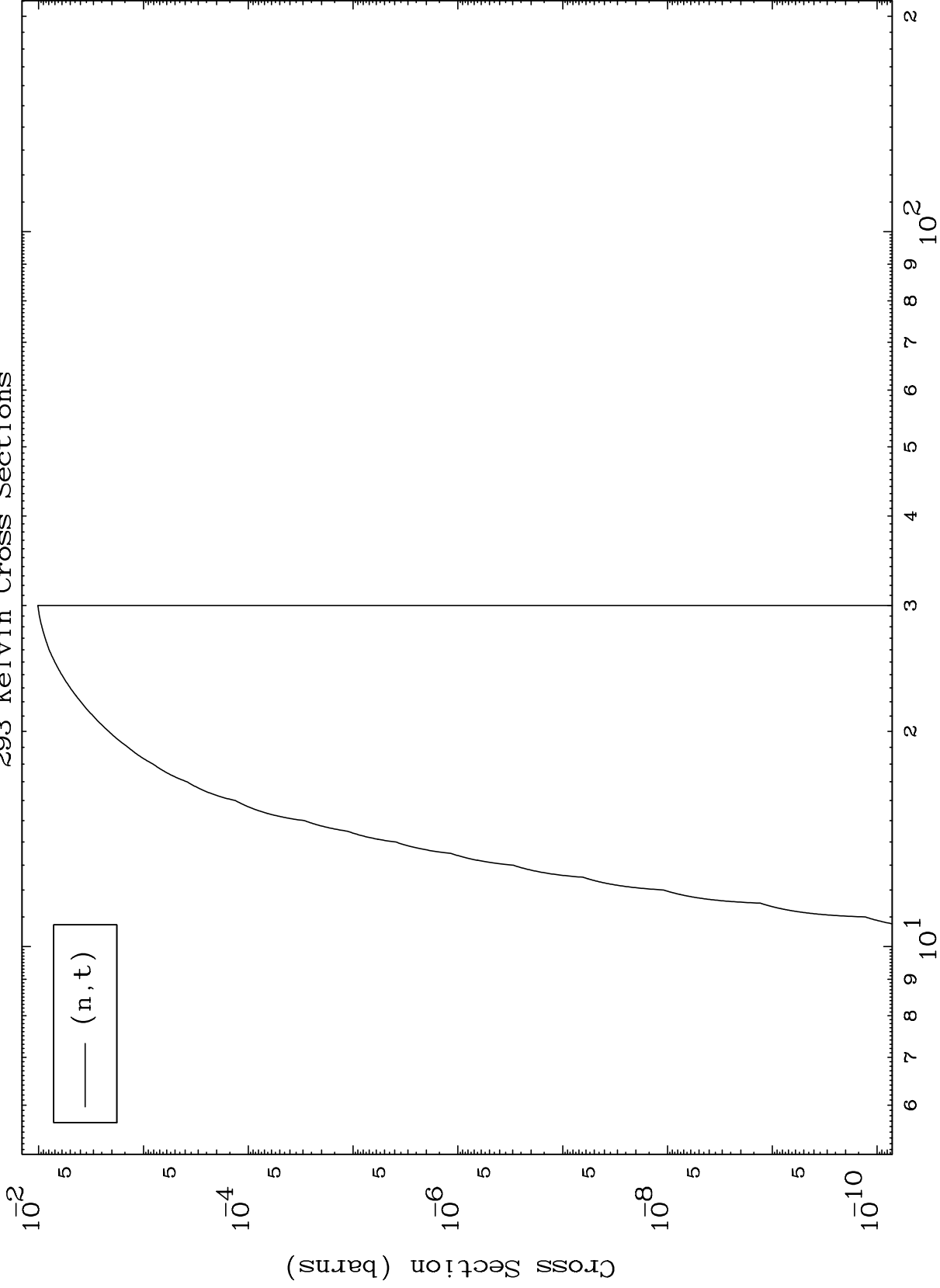
Incident Energy (MeV)

80-Hg-188

MAT 8001

(n,t) Levels
293 Kelvin Cross Sections

80-Hg-188



15

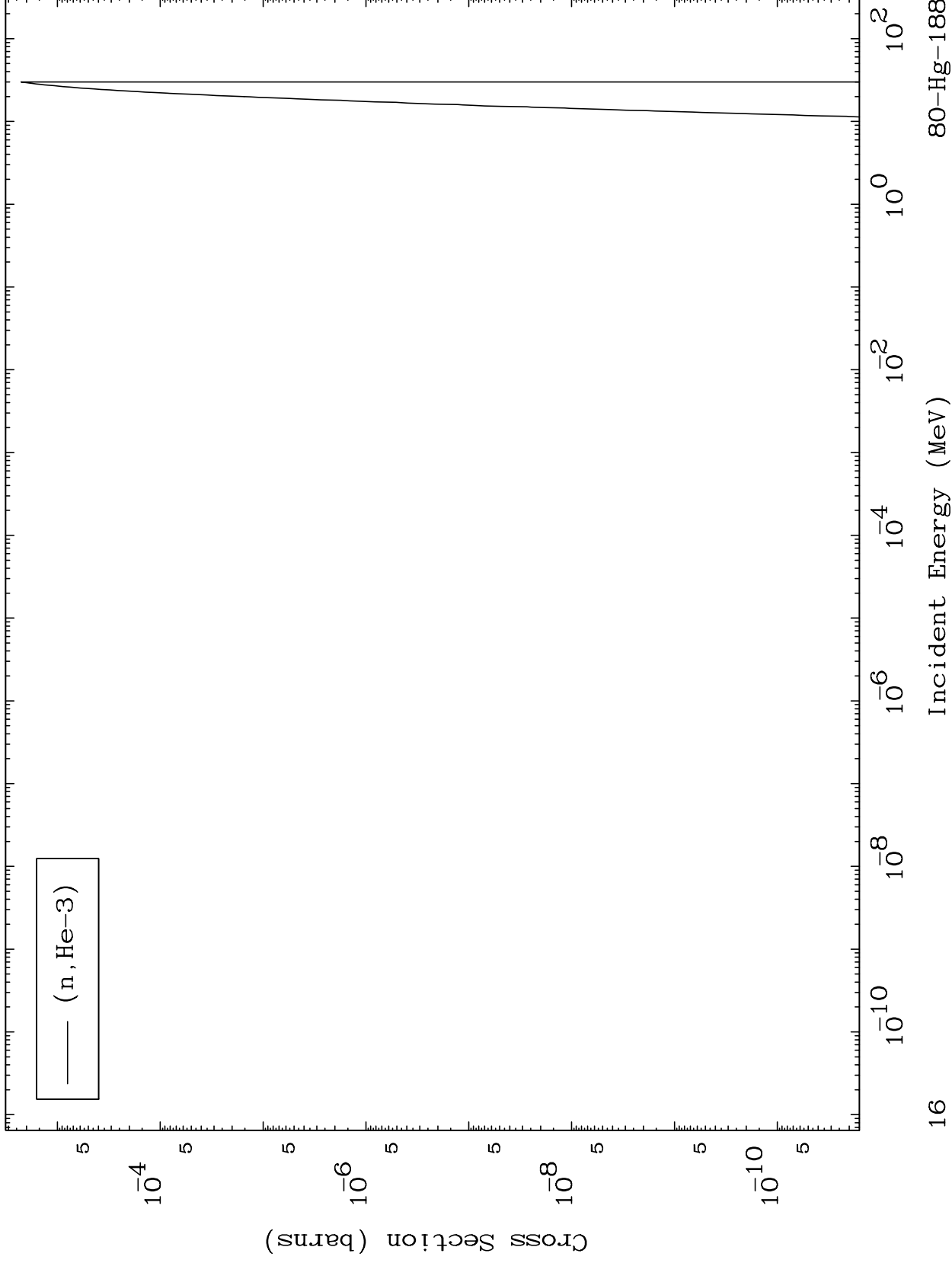
Incident Energy (MeV)

80-Hg-188

MAT 8001

(n,He3) Levels
293 Kelvin Cross Sections

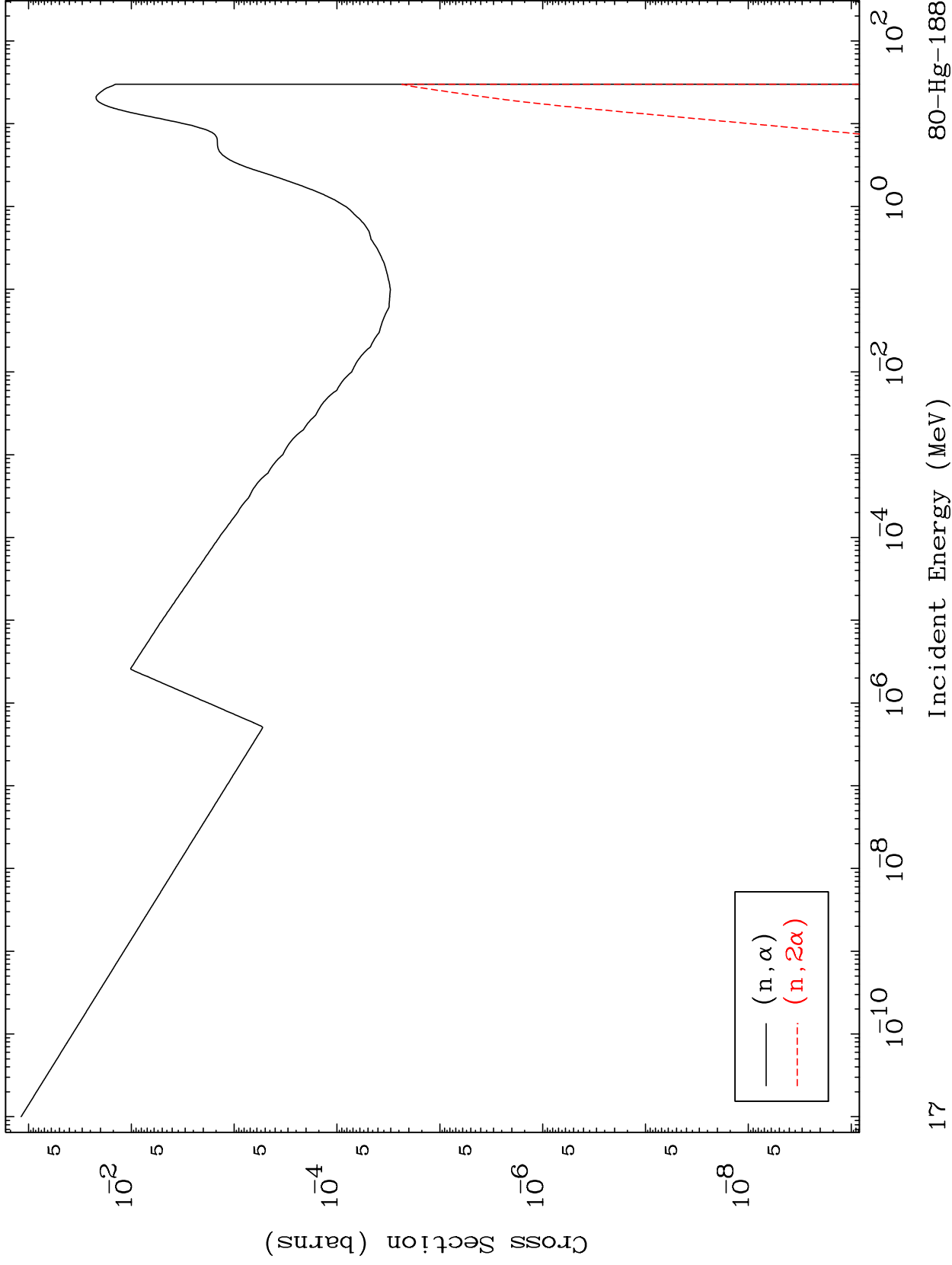
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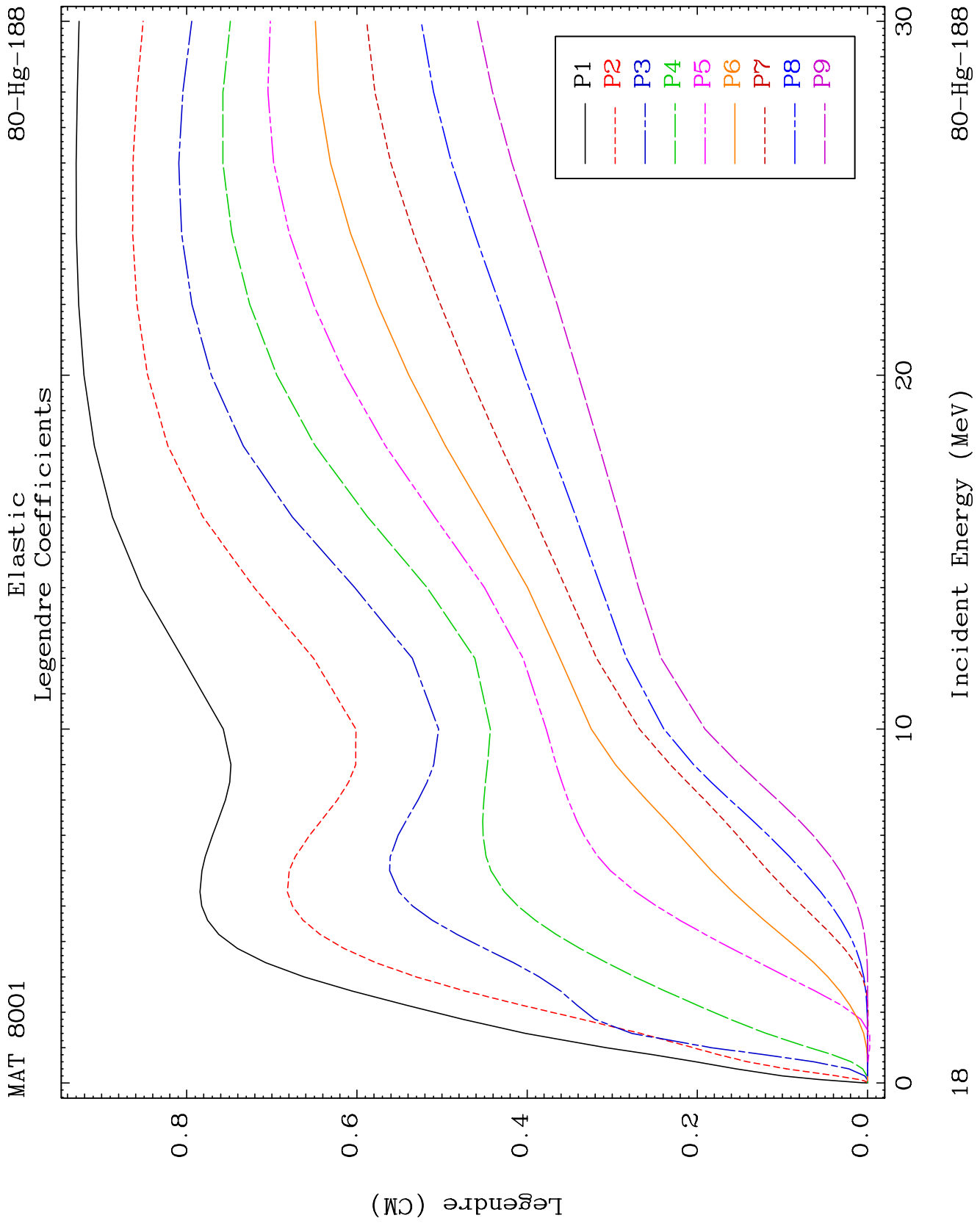


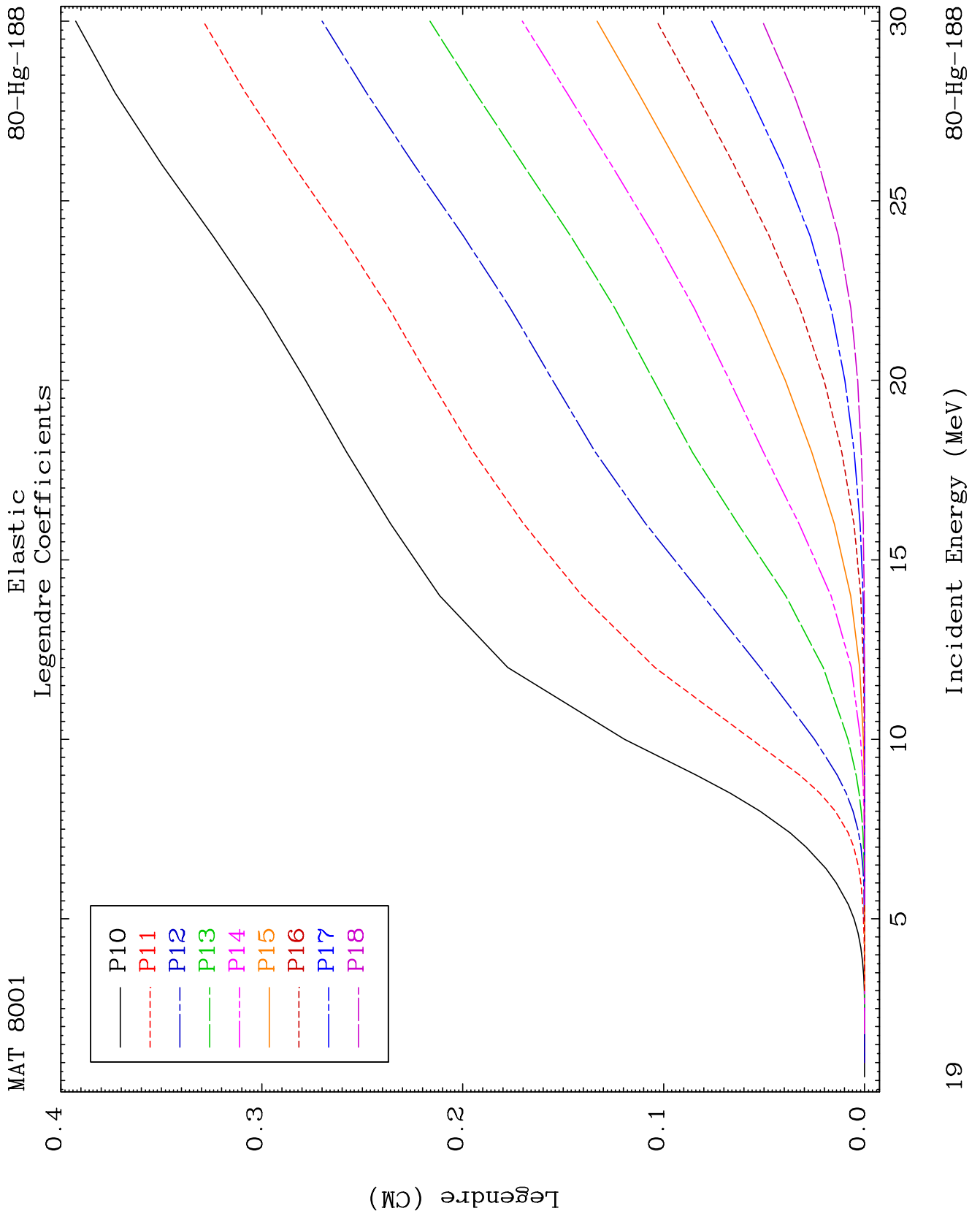
MAT 8001

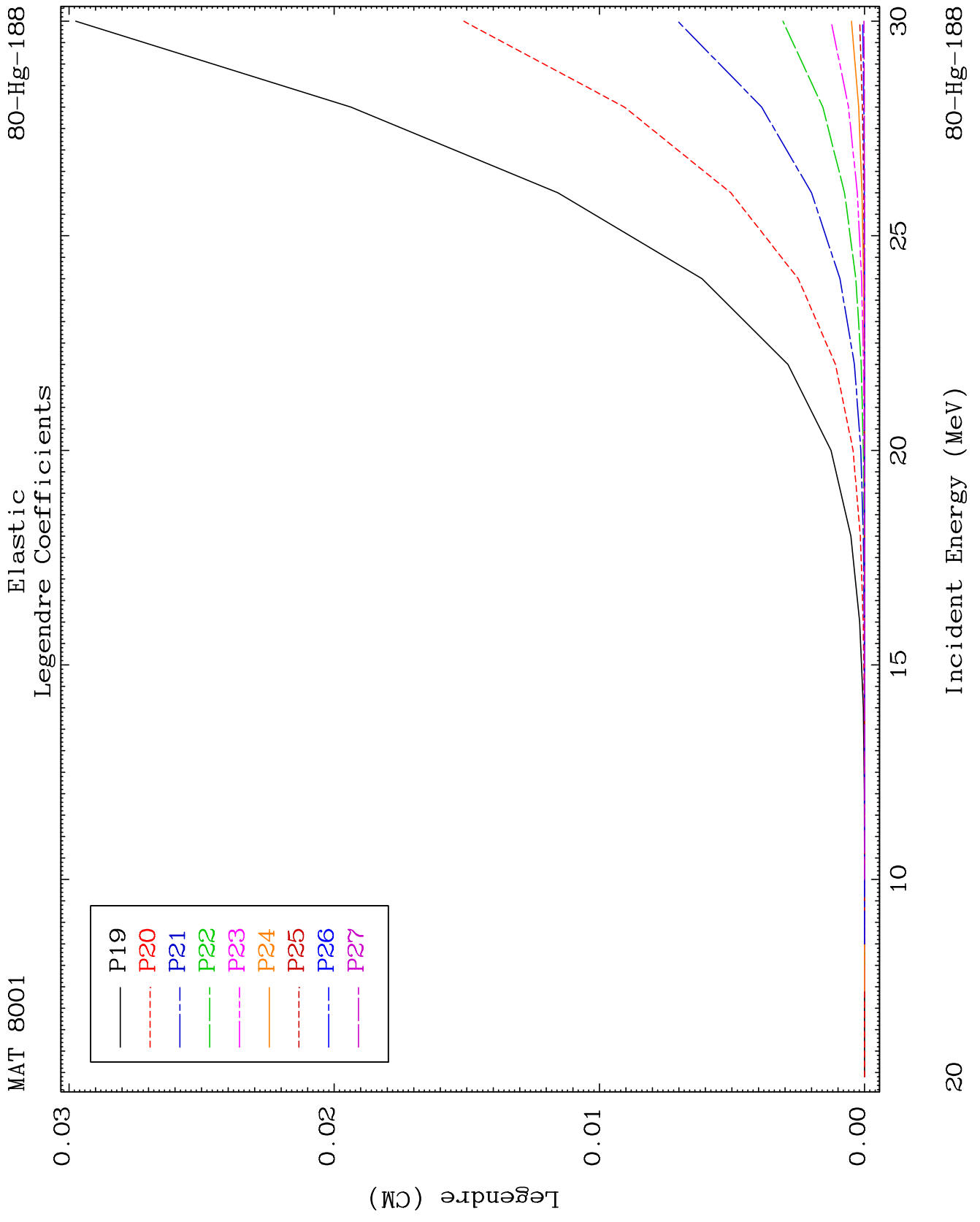
(n, α) Levels
293 Kelvin Cross Sections

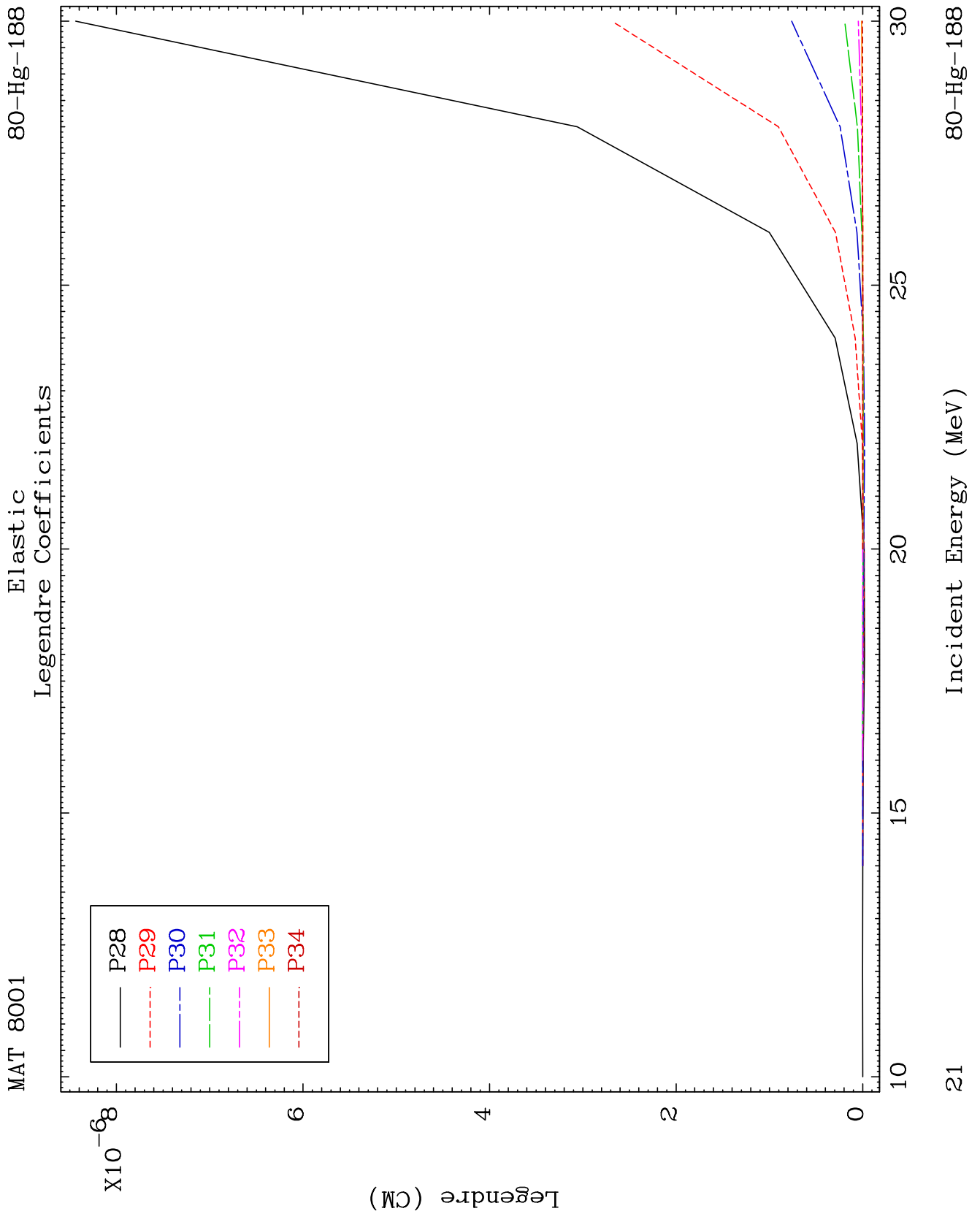
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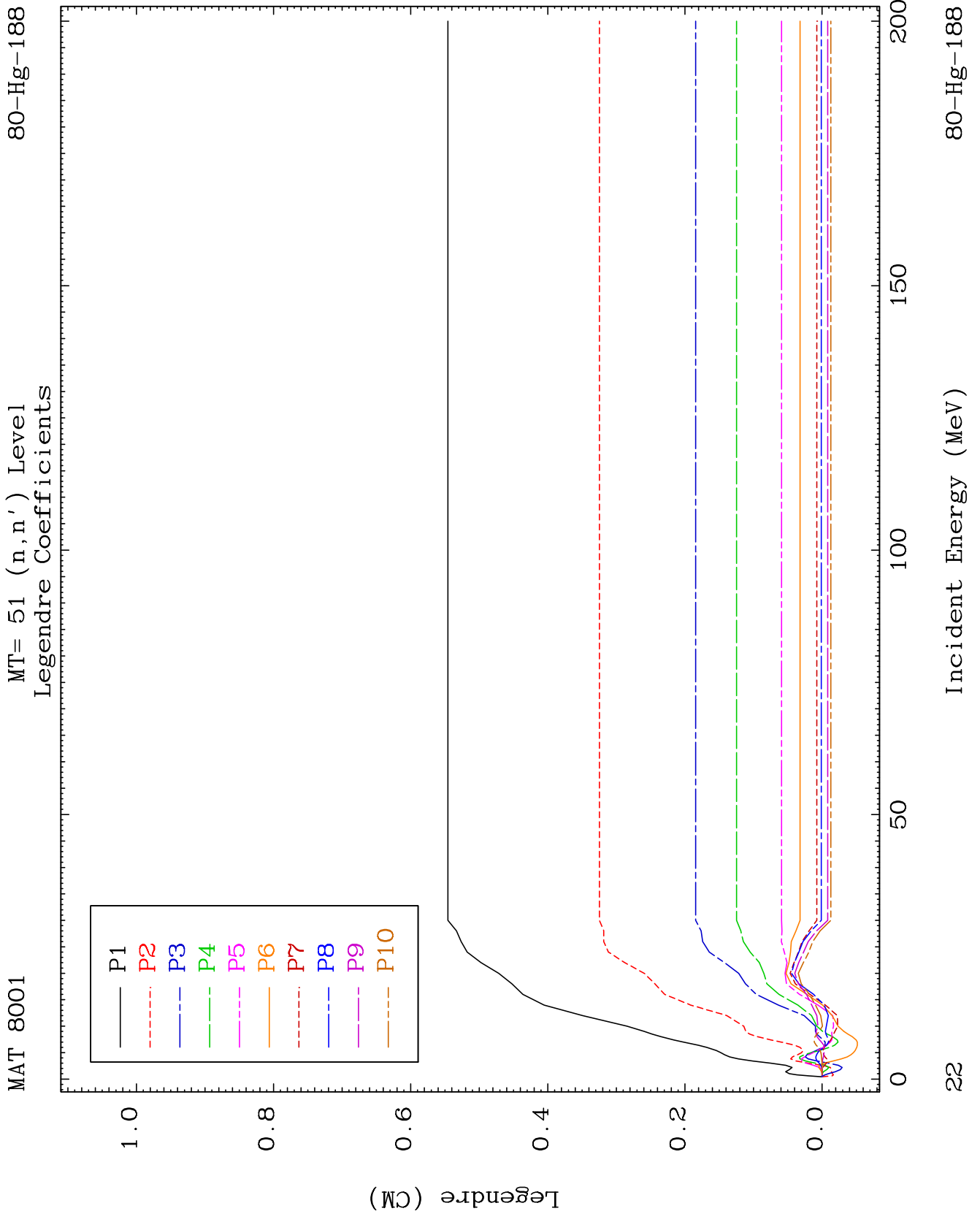


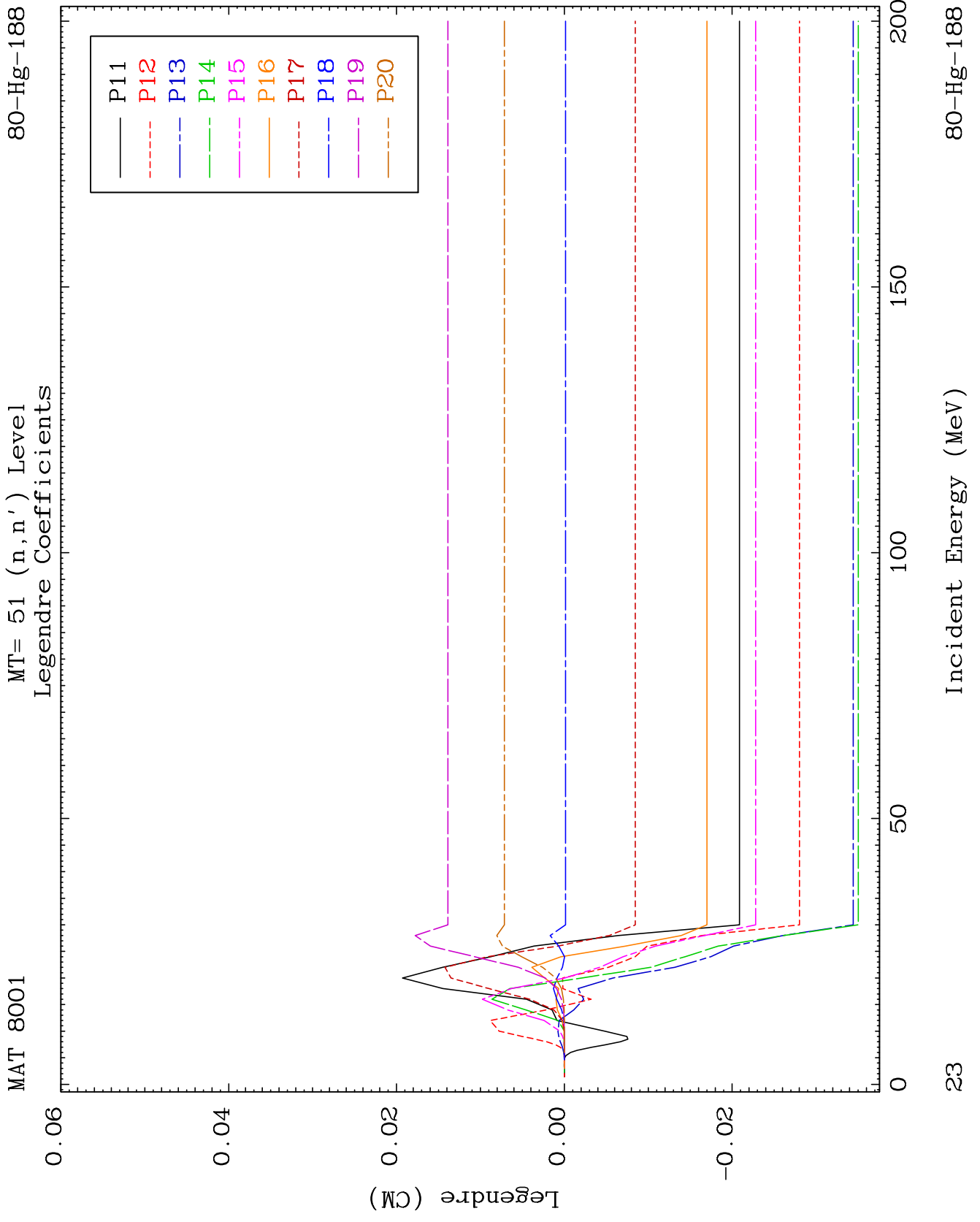








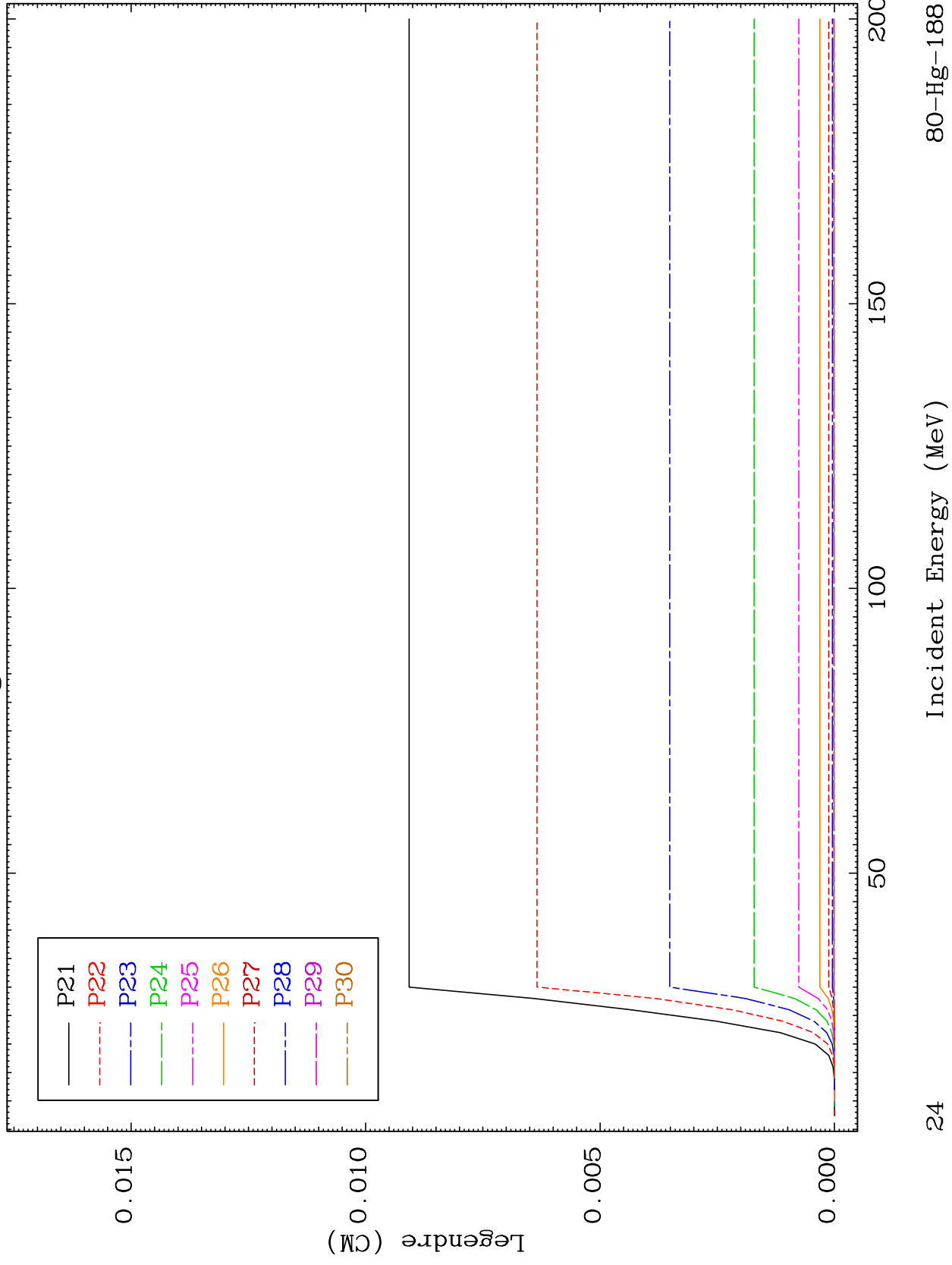




MAT 8001

MT= 51 (n,n') Level
Legendre Coefficients

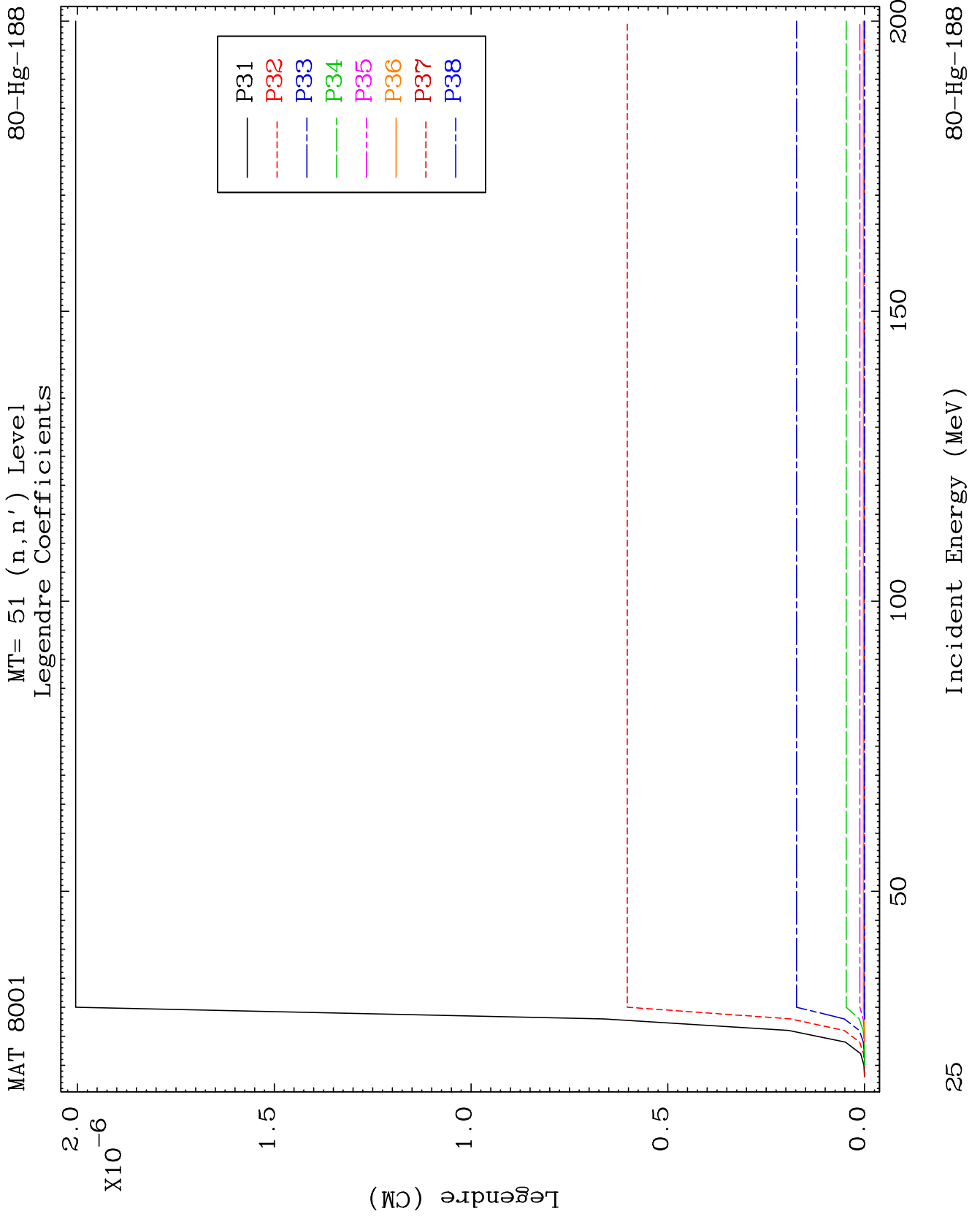
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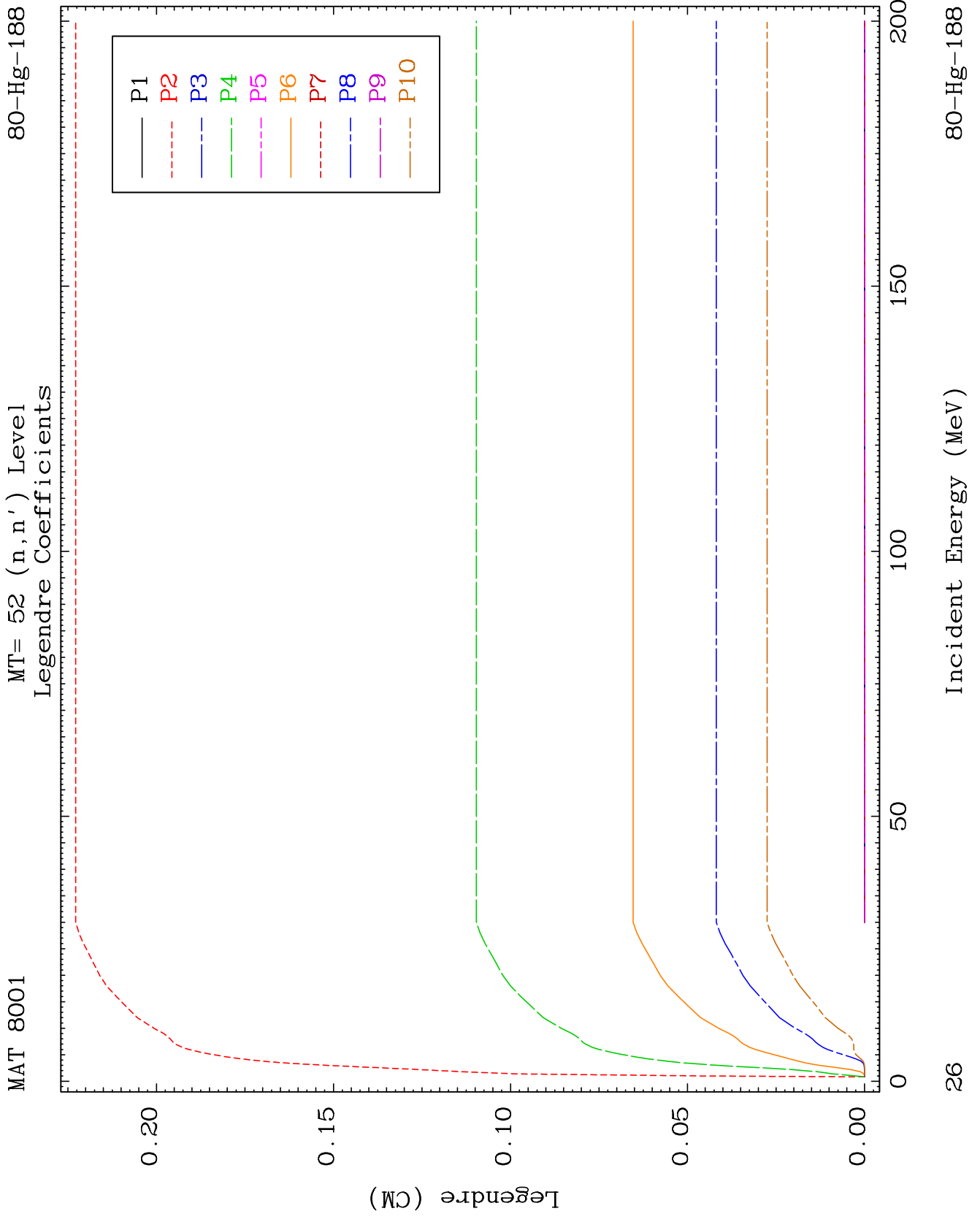


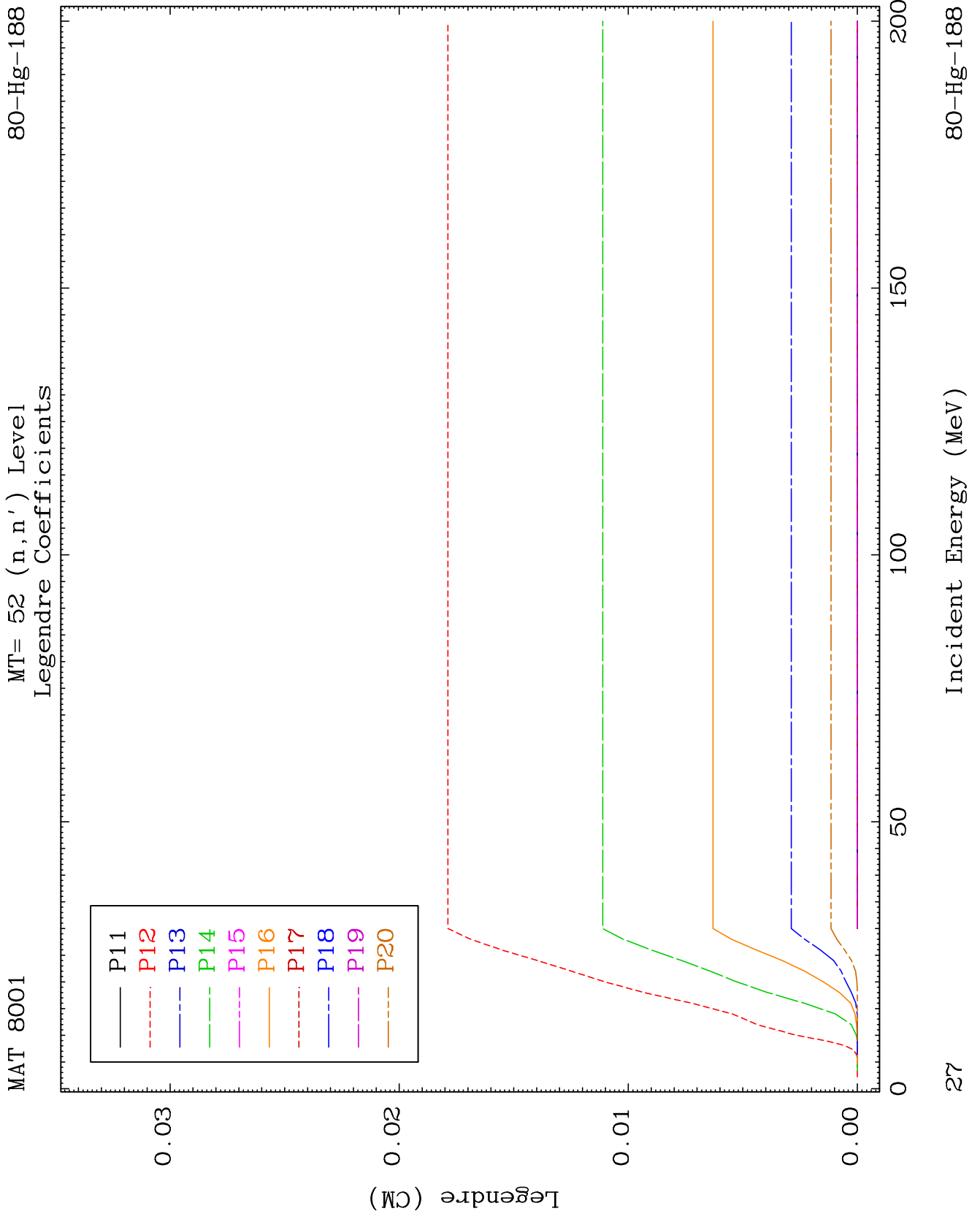
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Incident Energy (MeV)

80-Hg-188



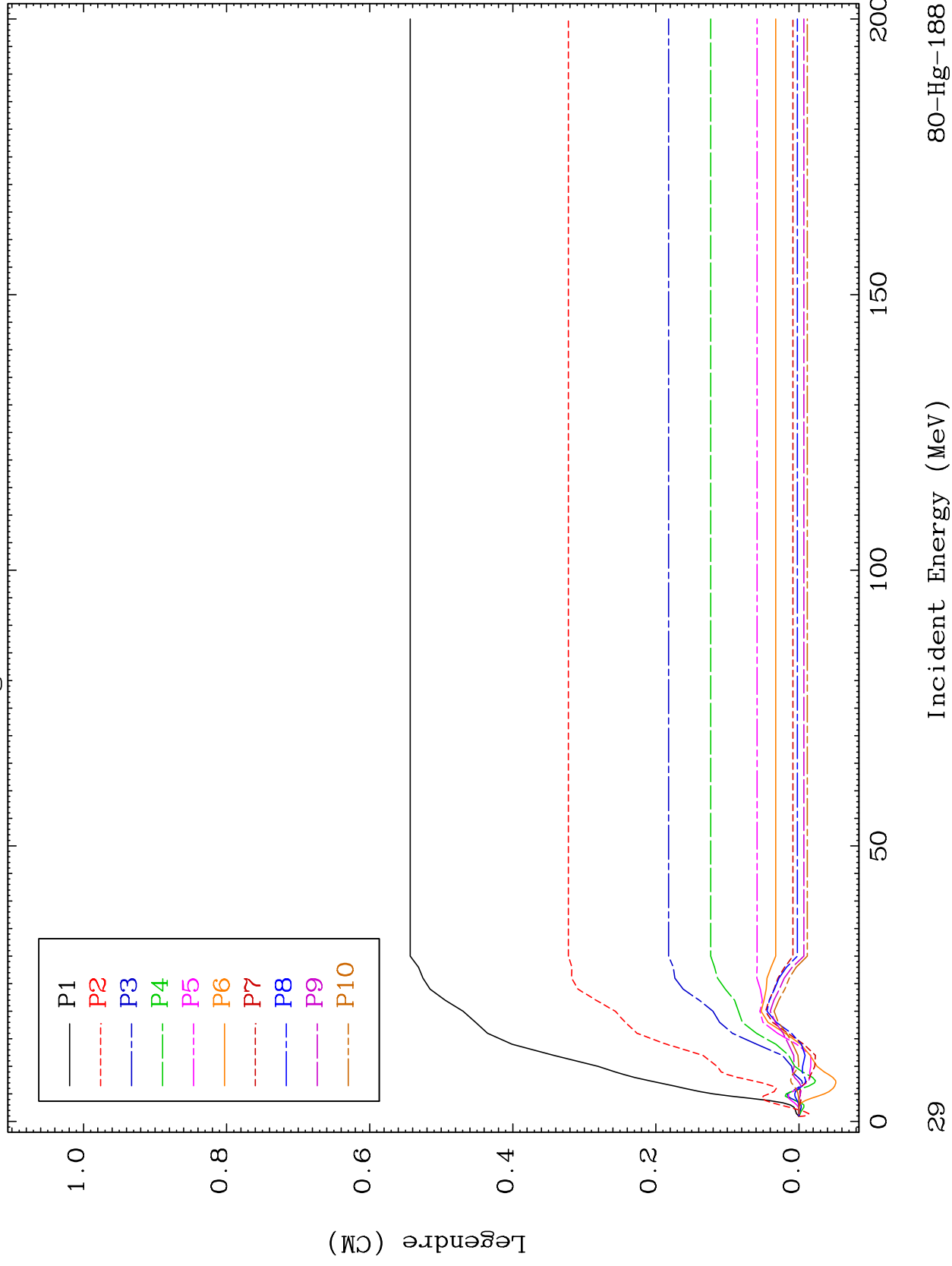




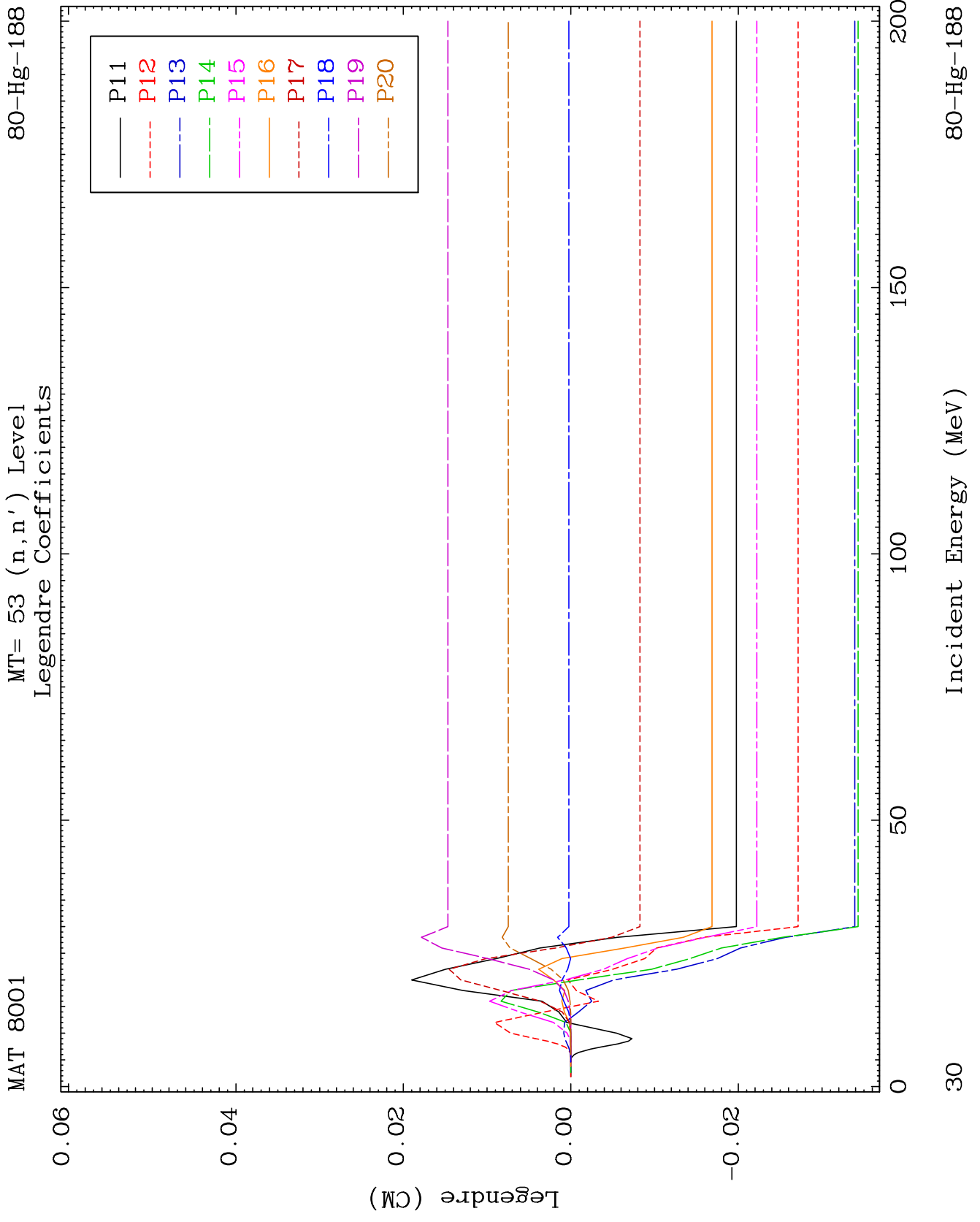
MAT 8001

MT= 53 (n,n') Level
Legendre Coefficients

80-Hg-188



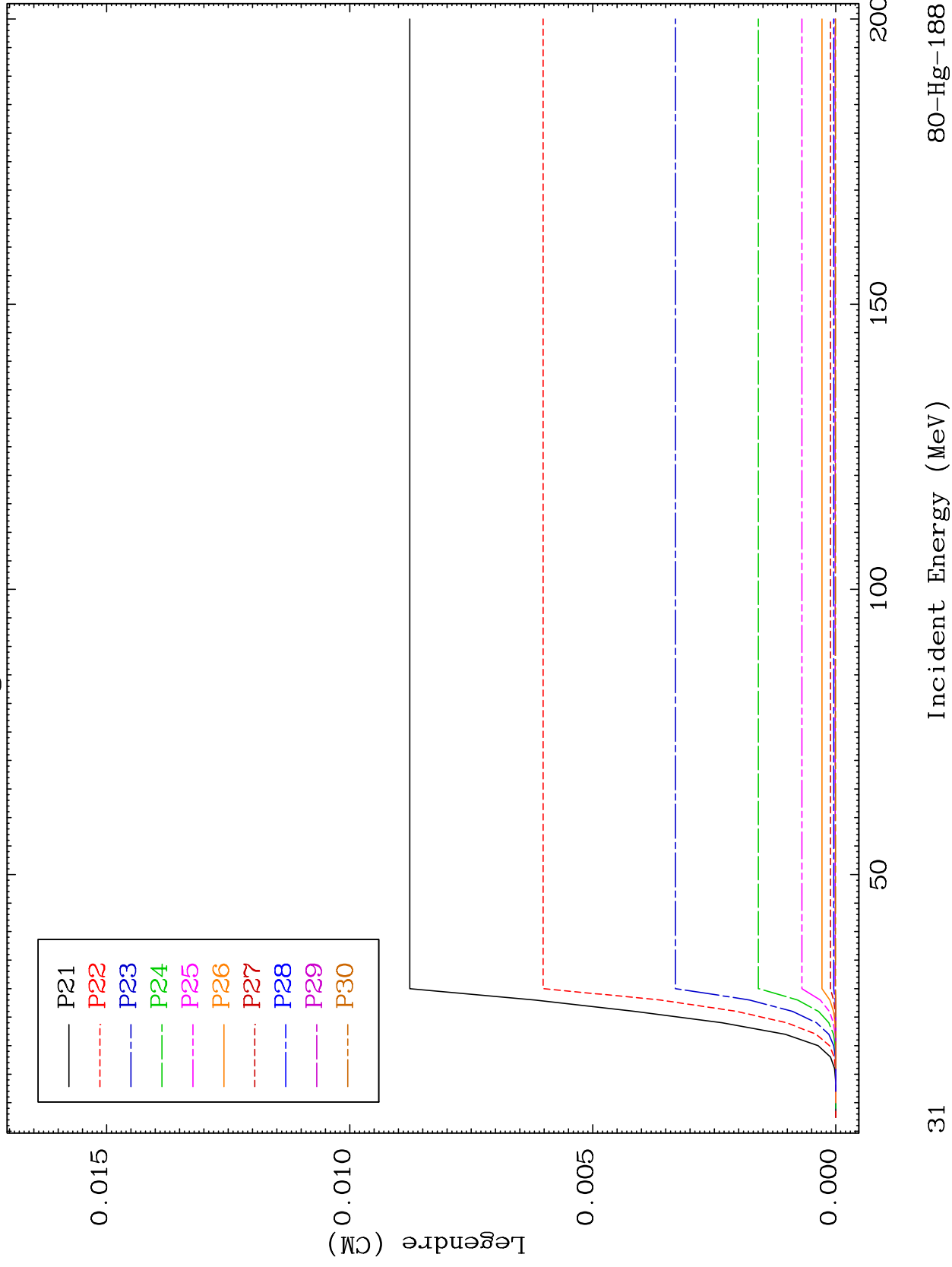
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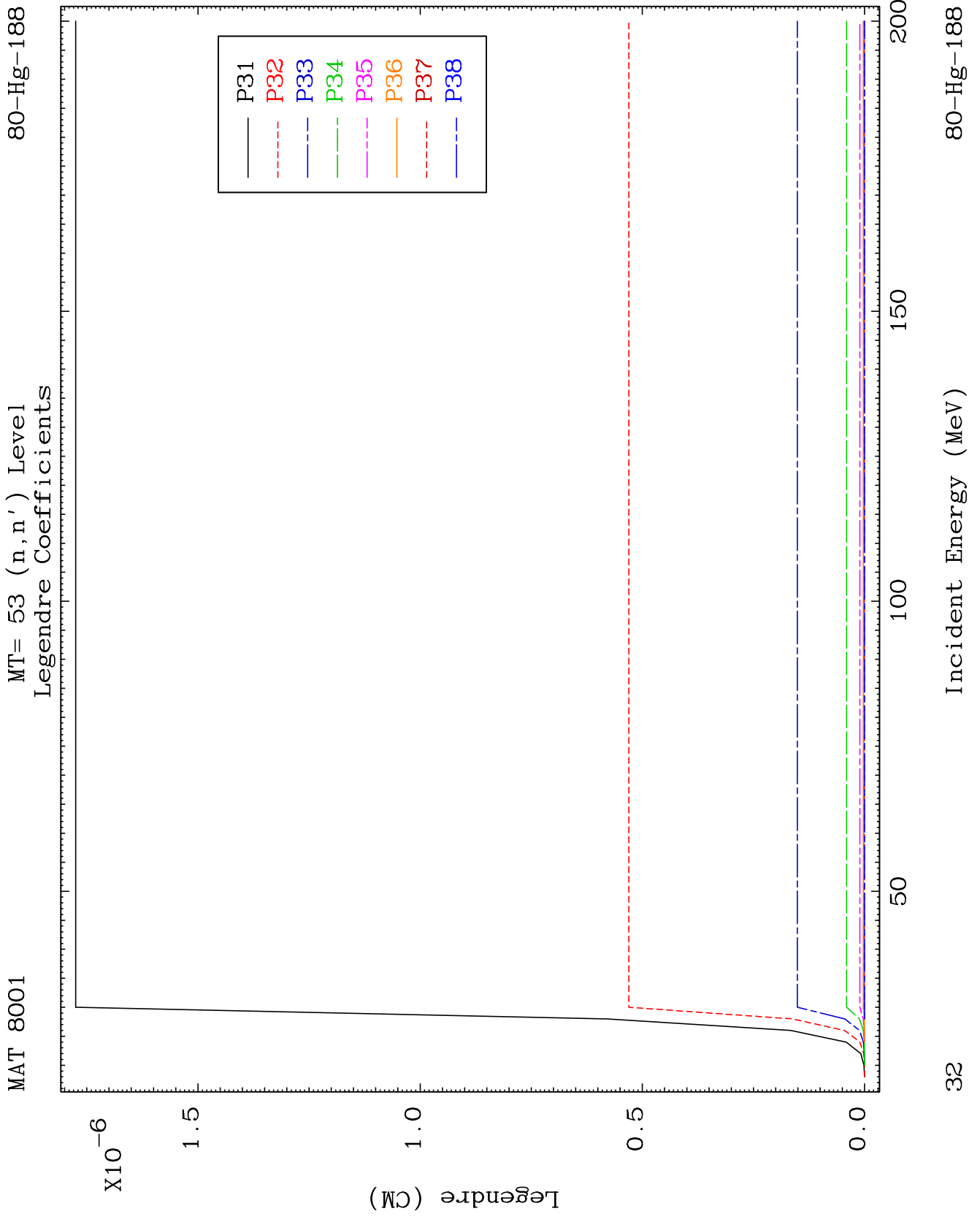


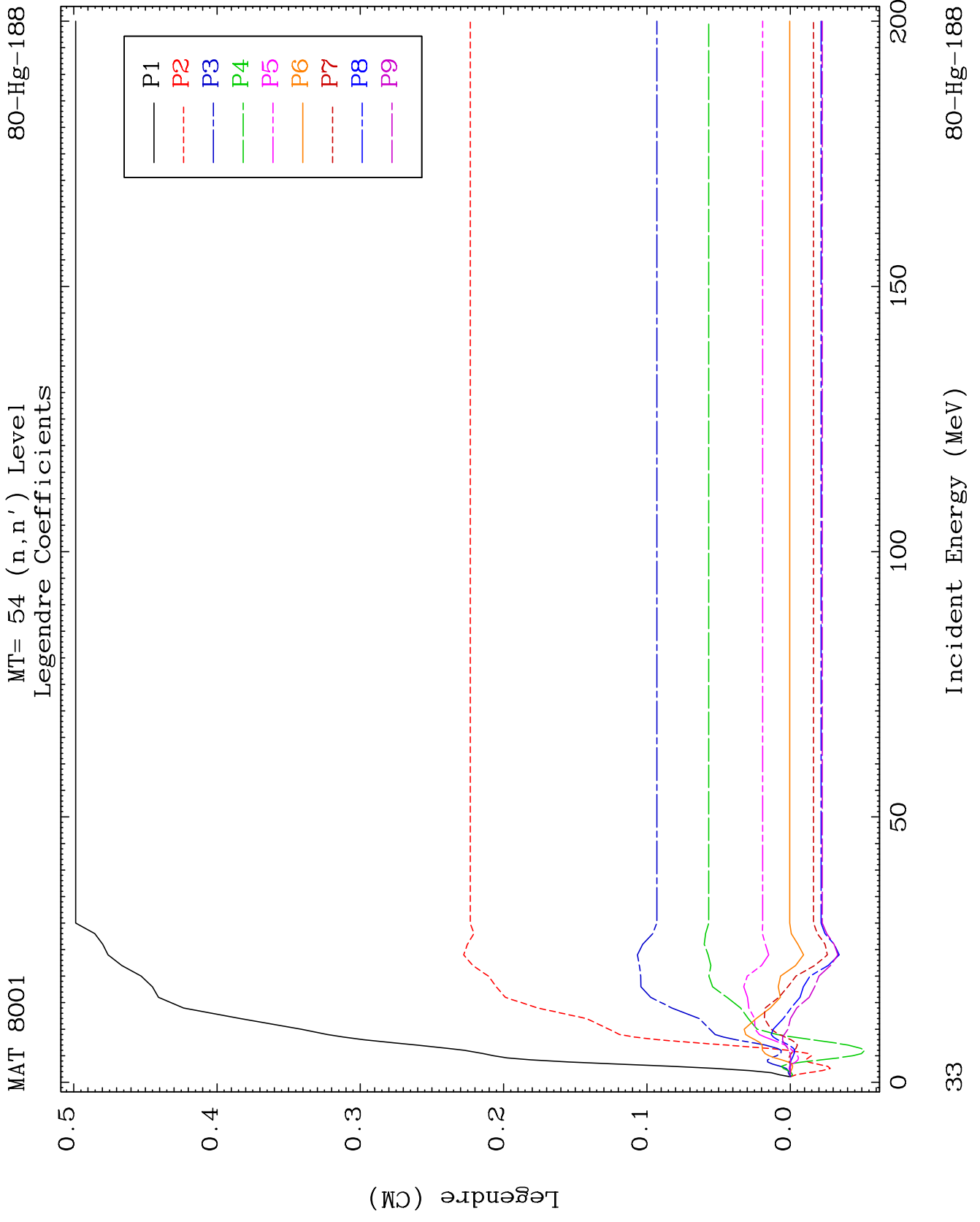
MAT 8001

MT= 53 (n,n') Level
Legendre Coefficients

80-Hg-188



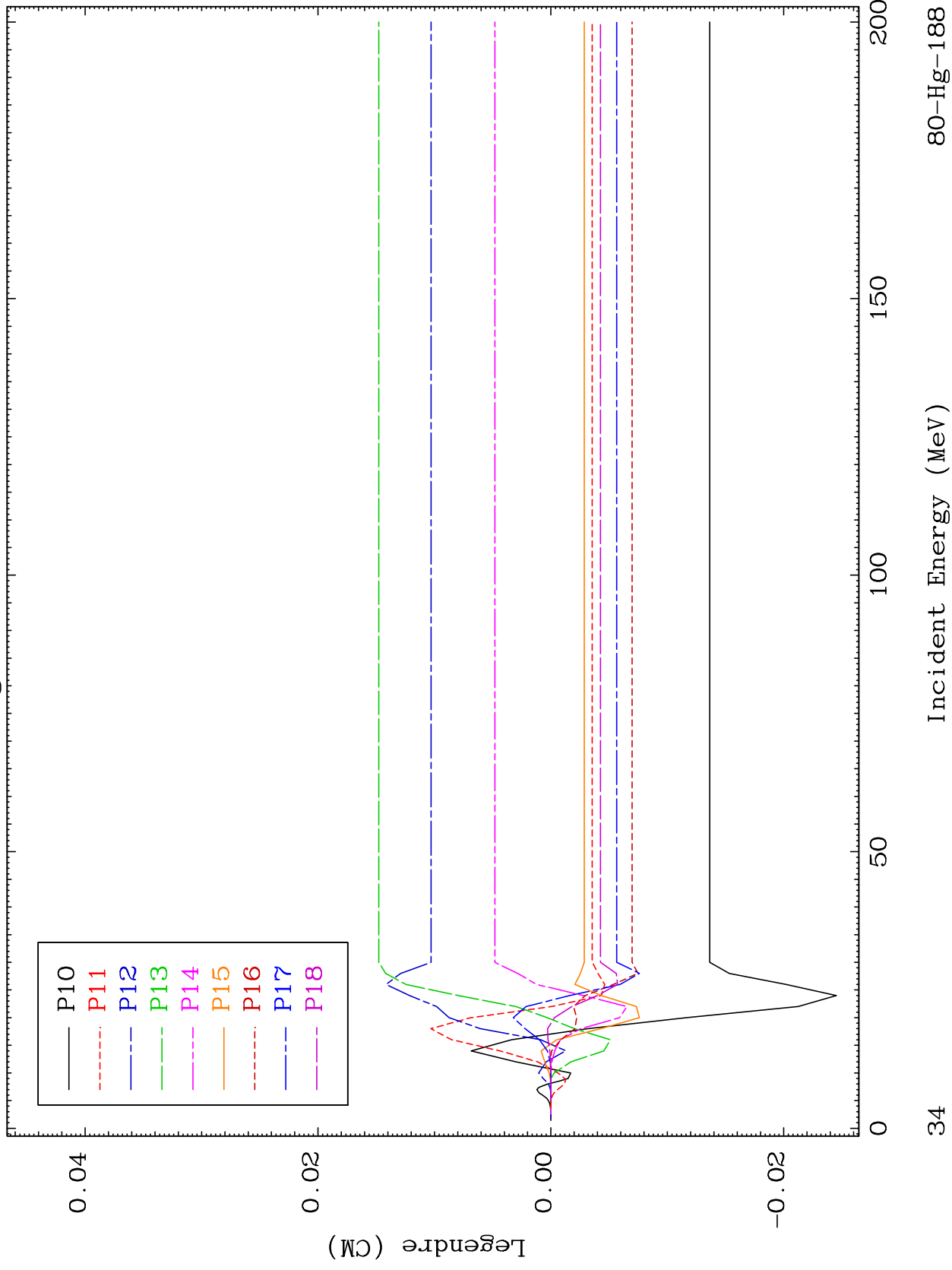




MAT 8001

MT= 54 (n,n') Level
Legendre Coefficients

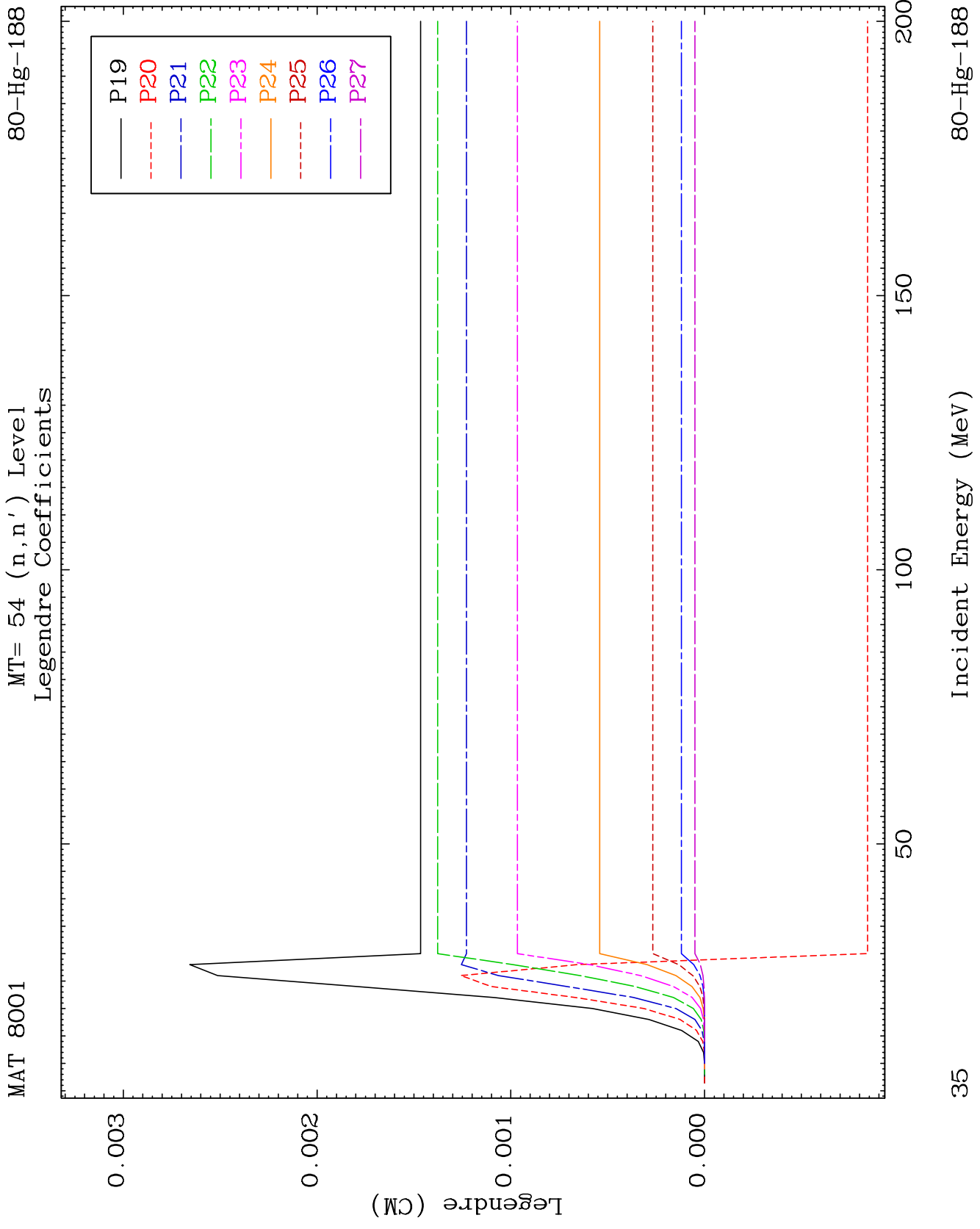
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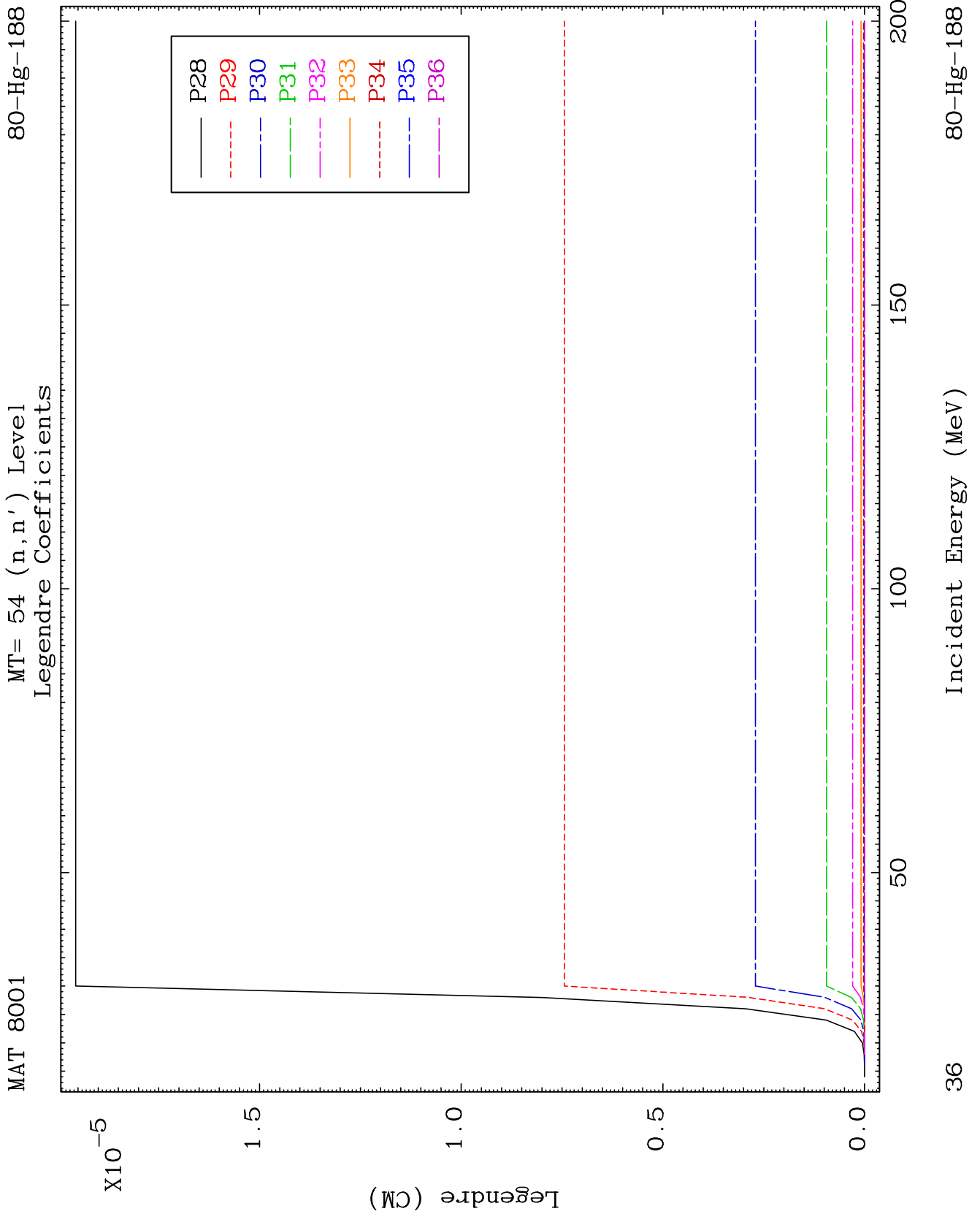


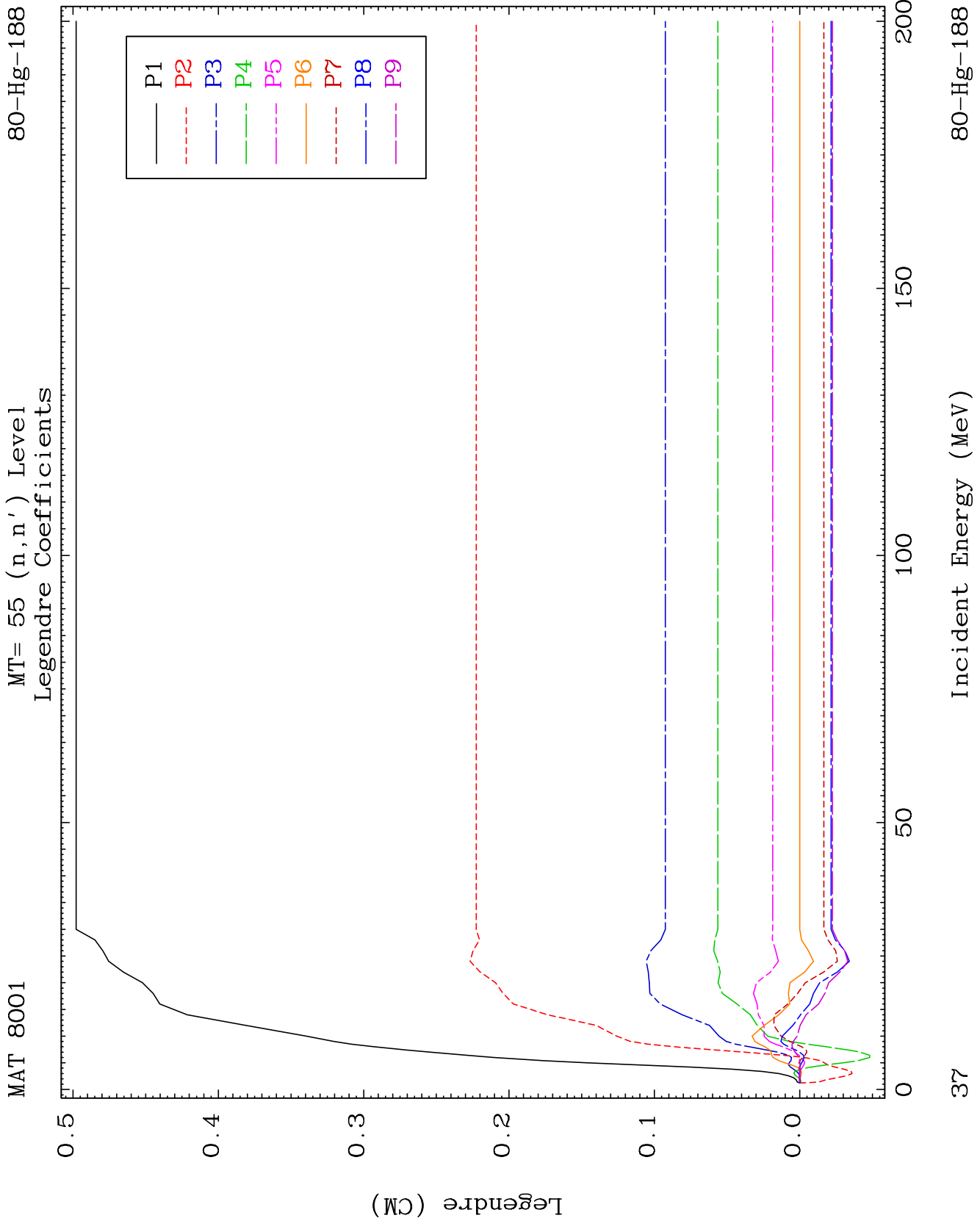
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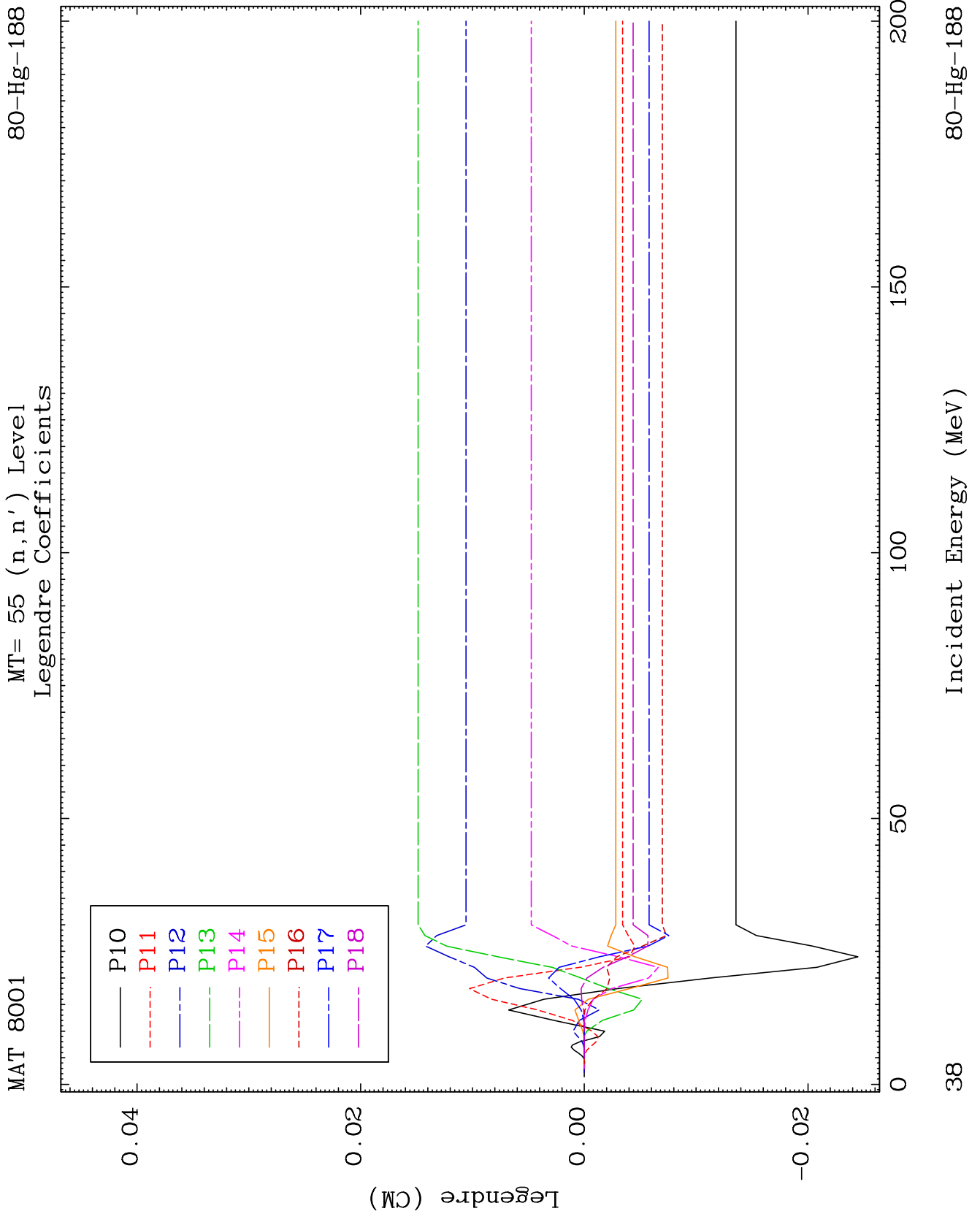
Incident Energy (MeV)

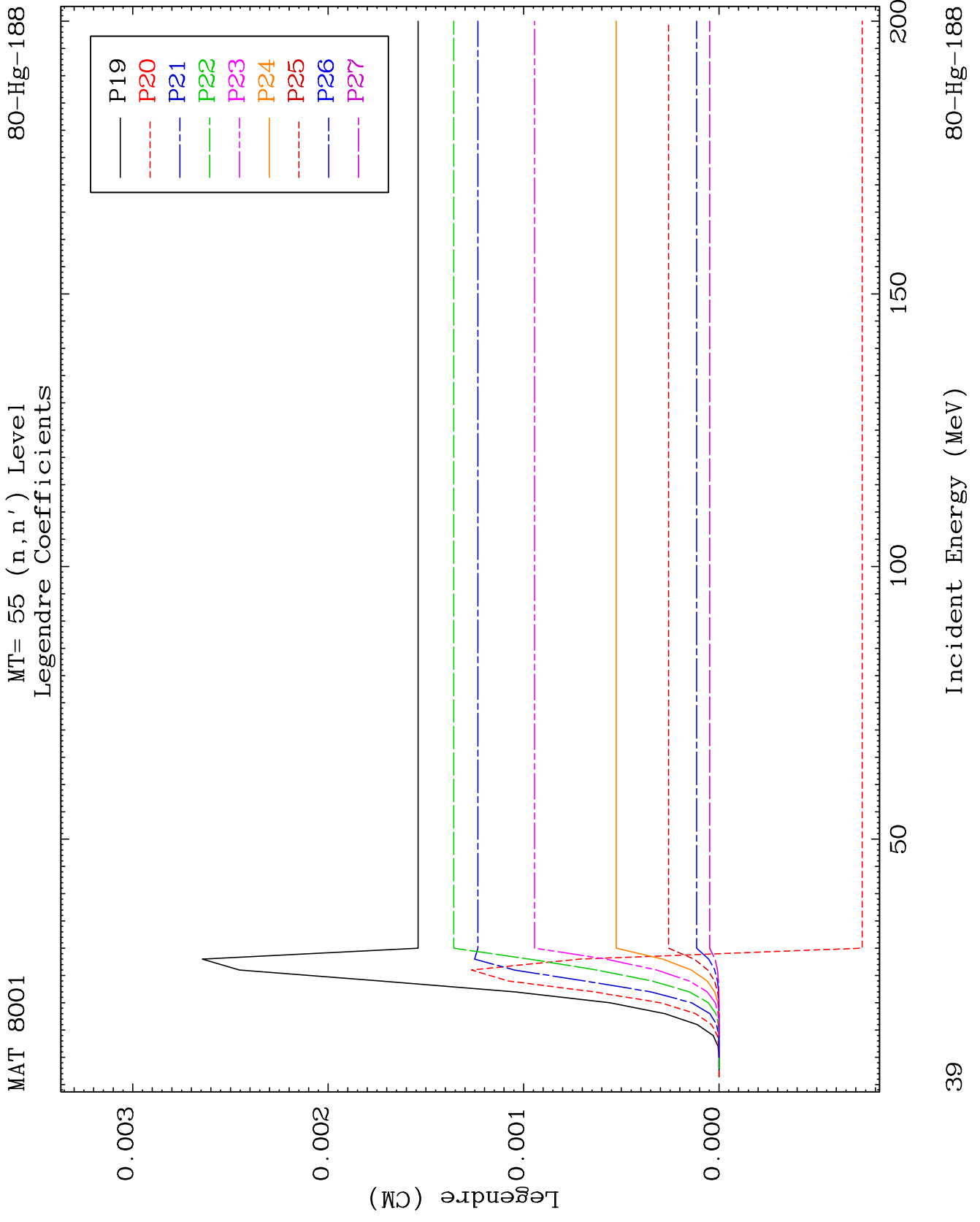
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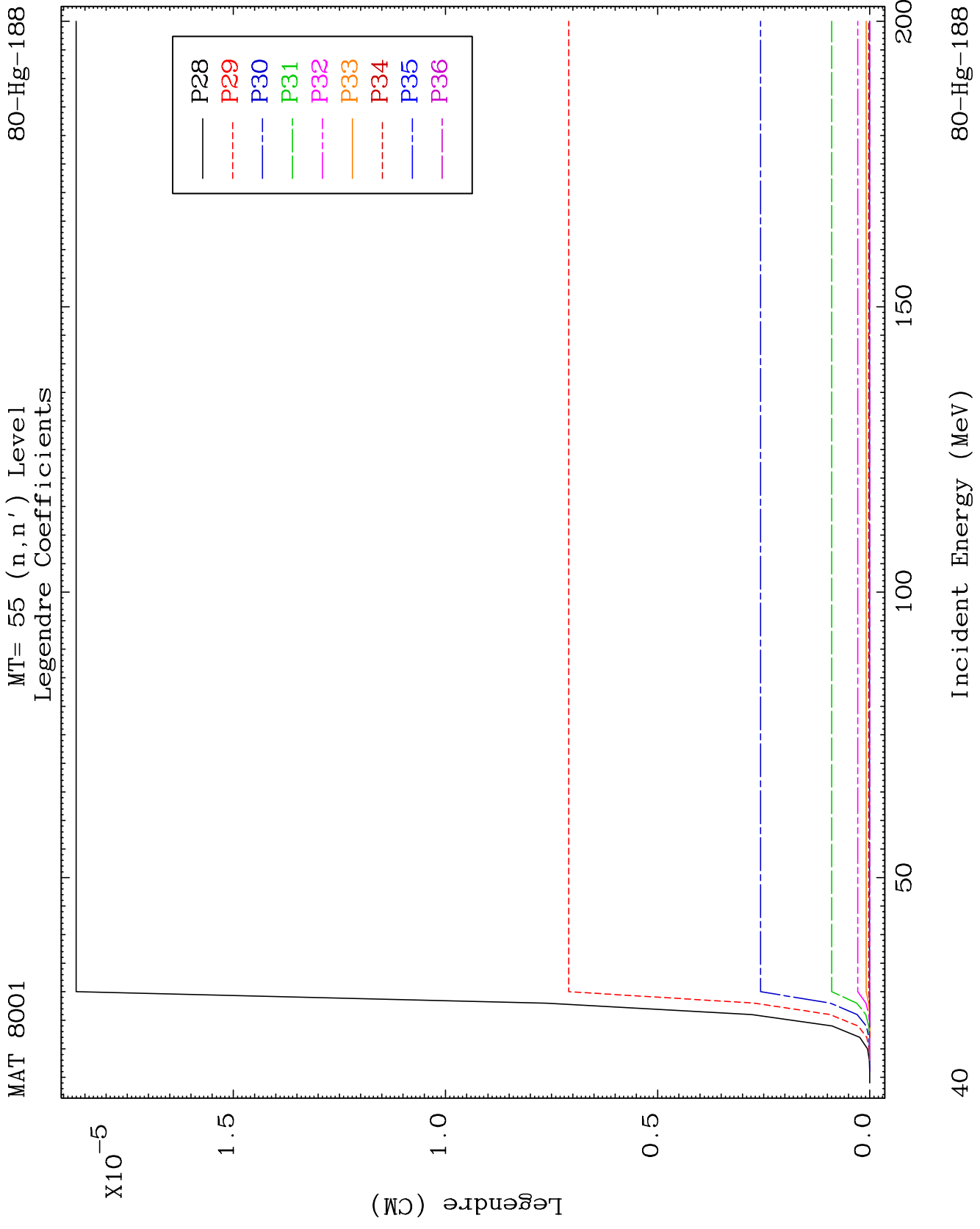


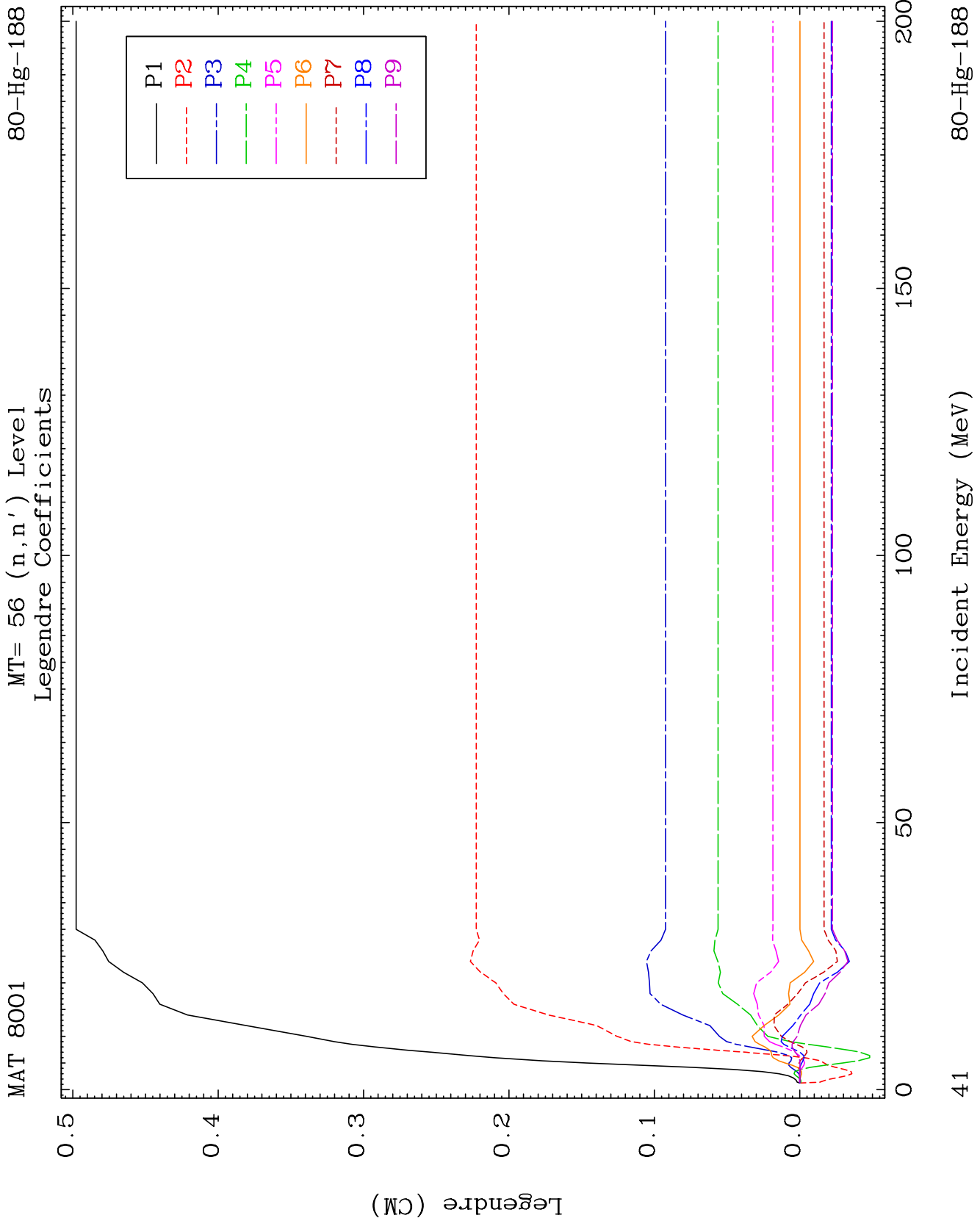


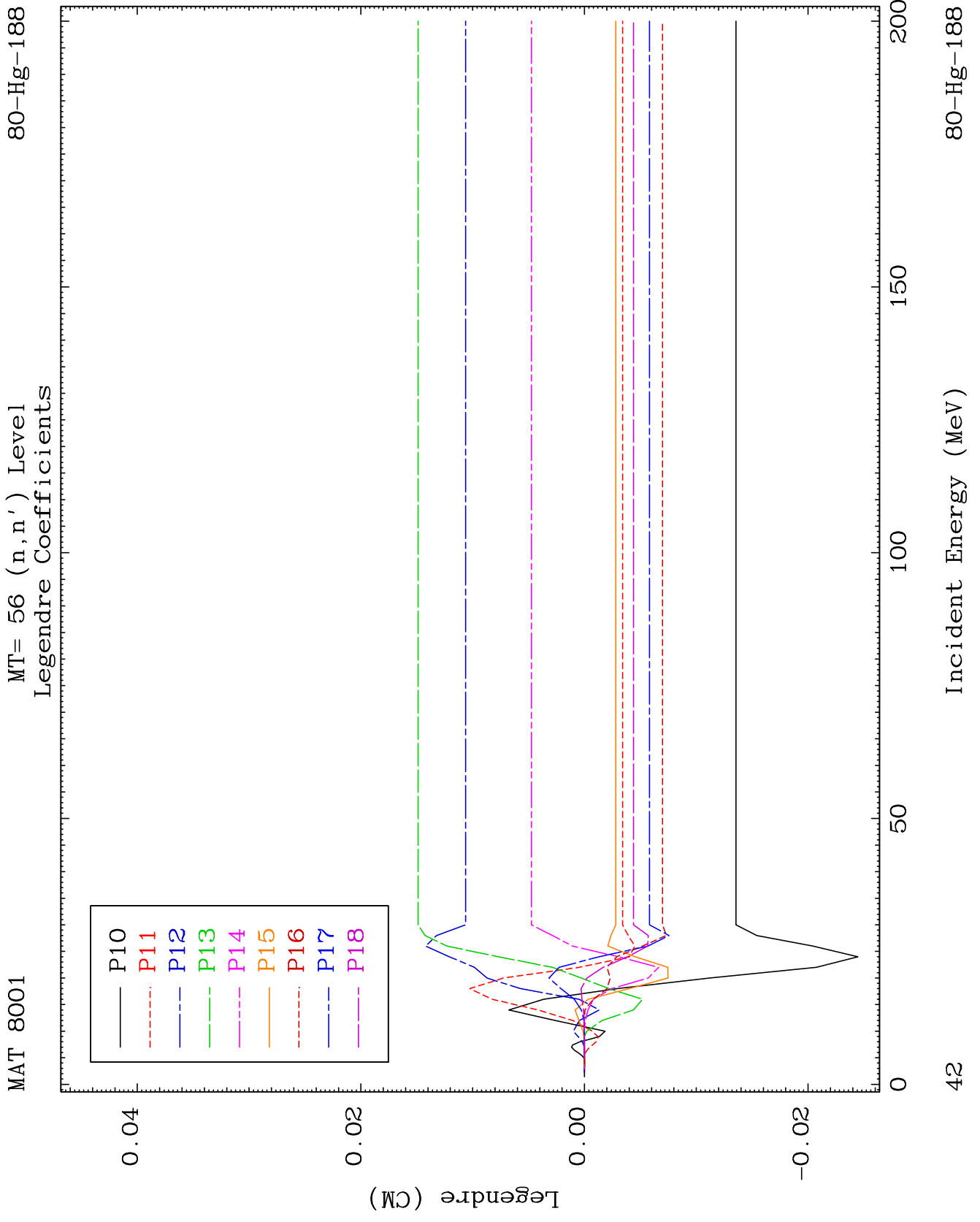


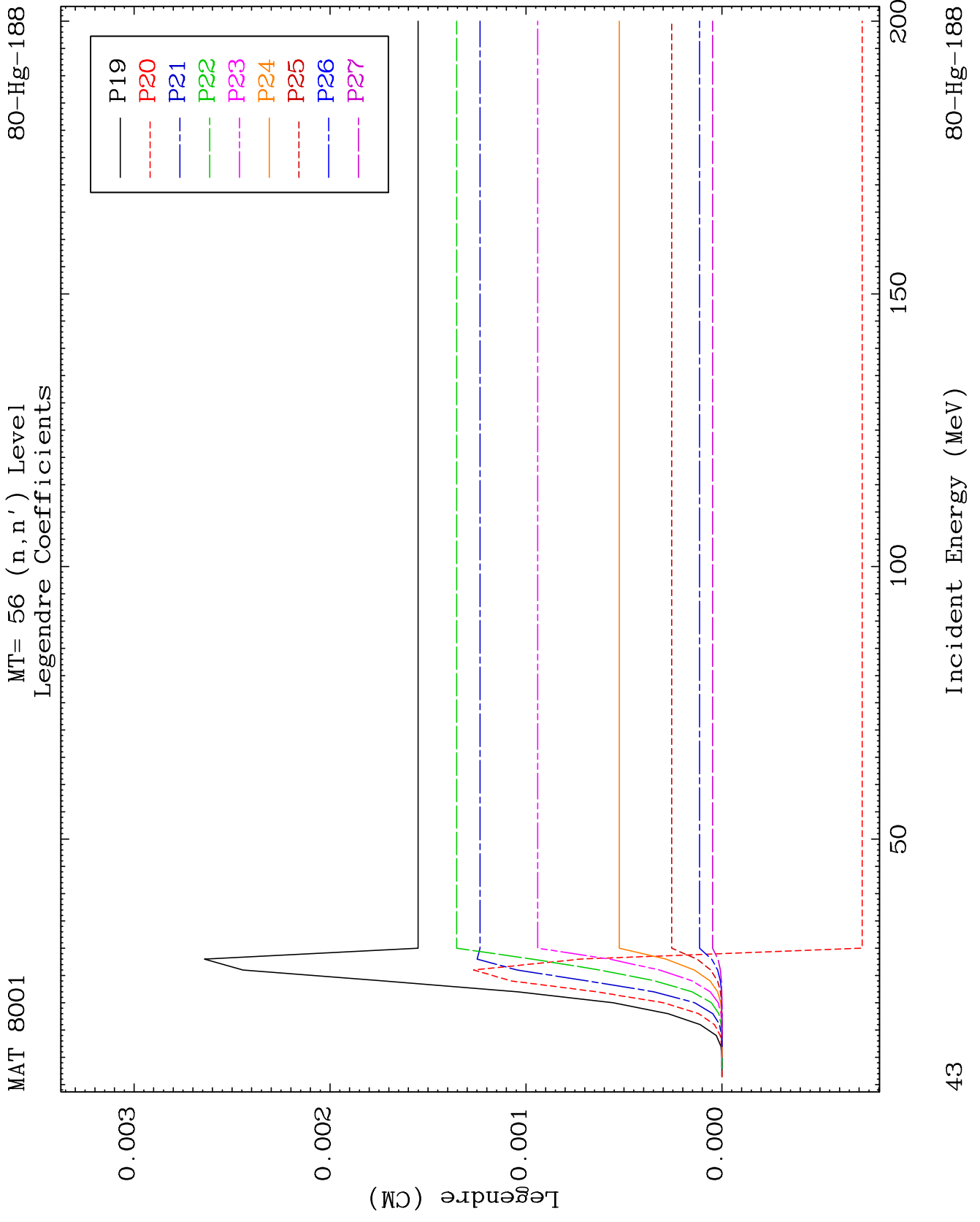


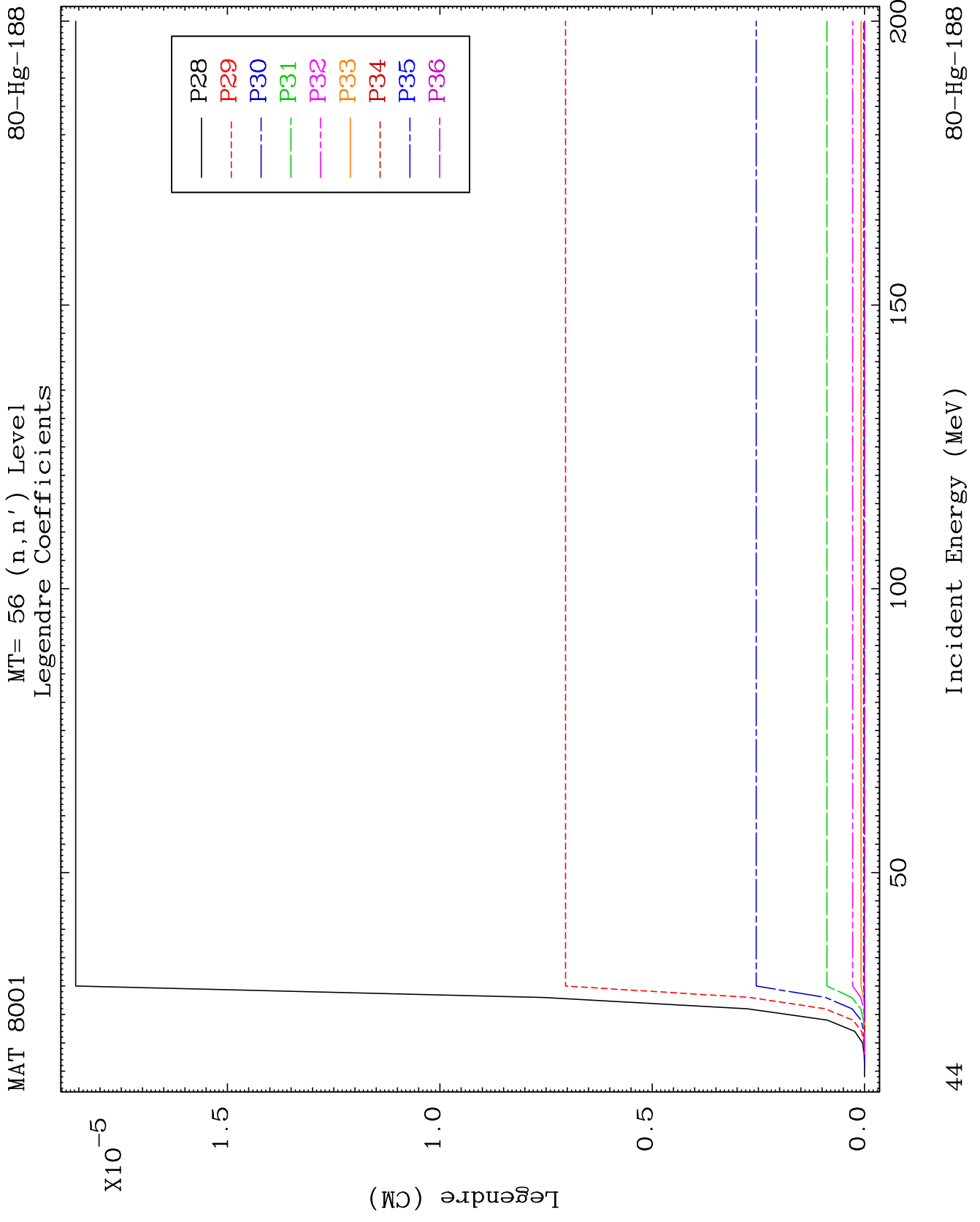


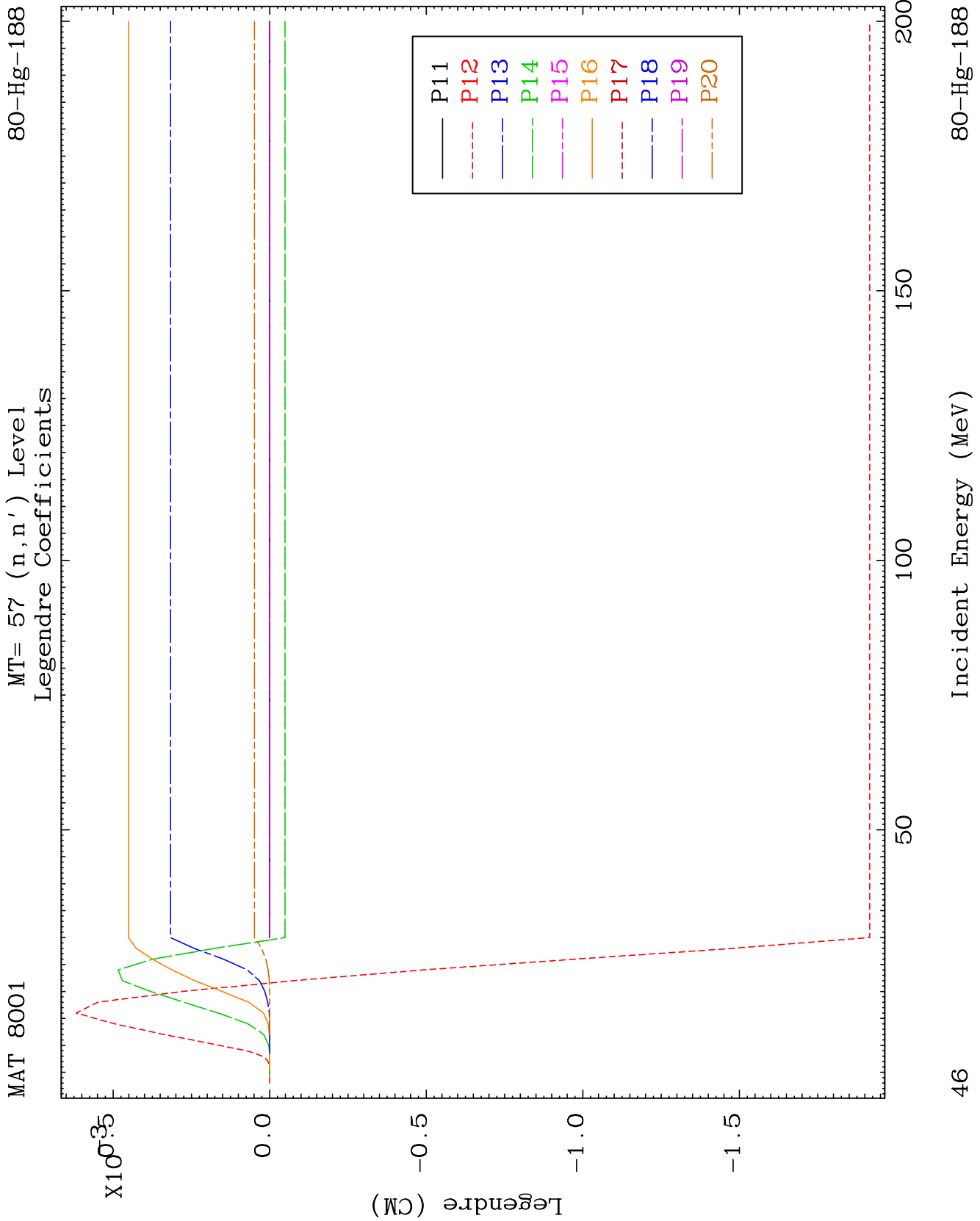


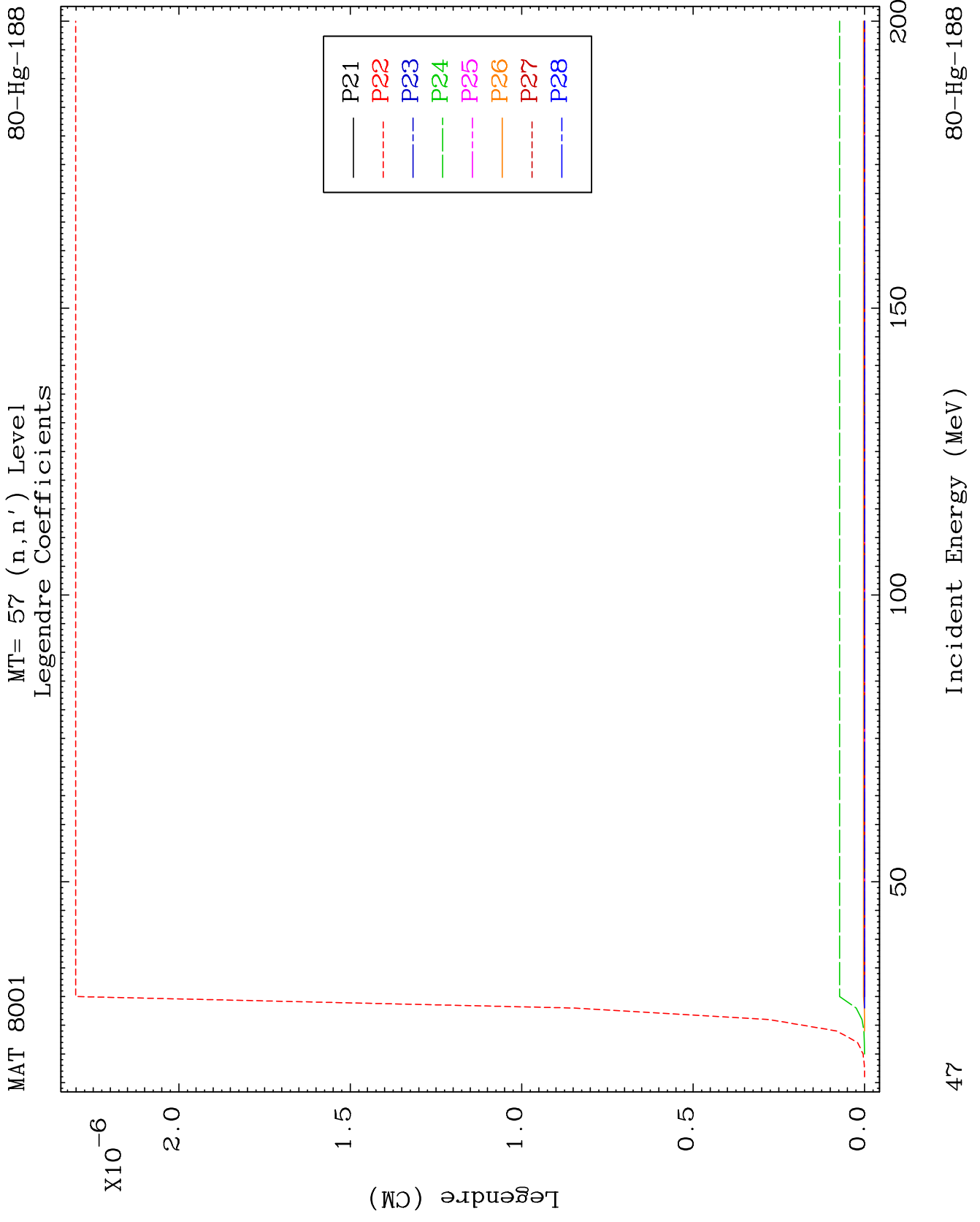


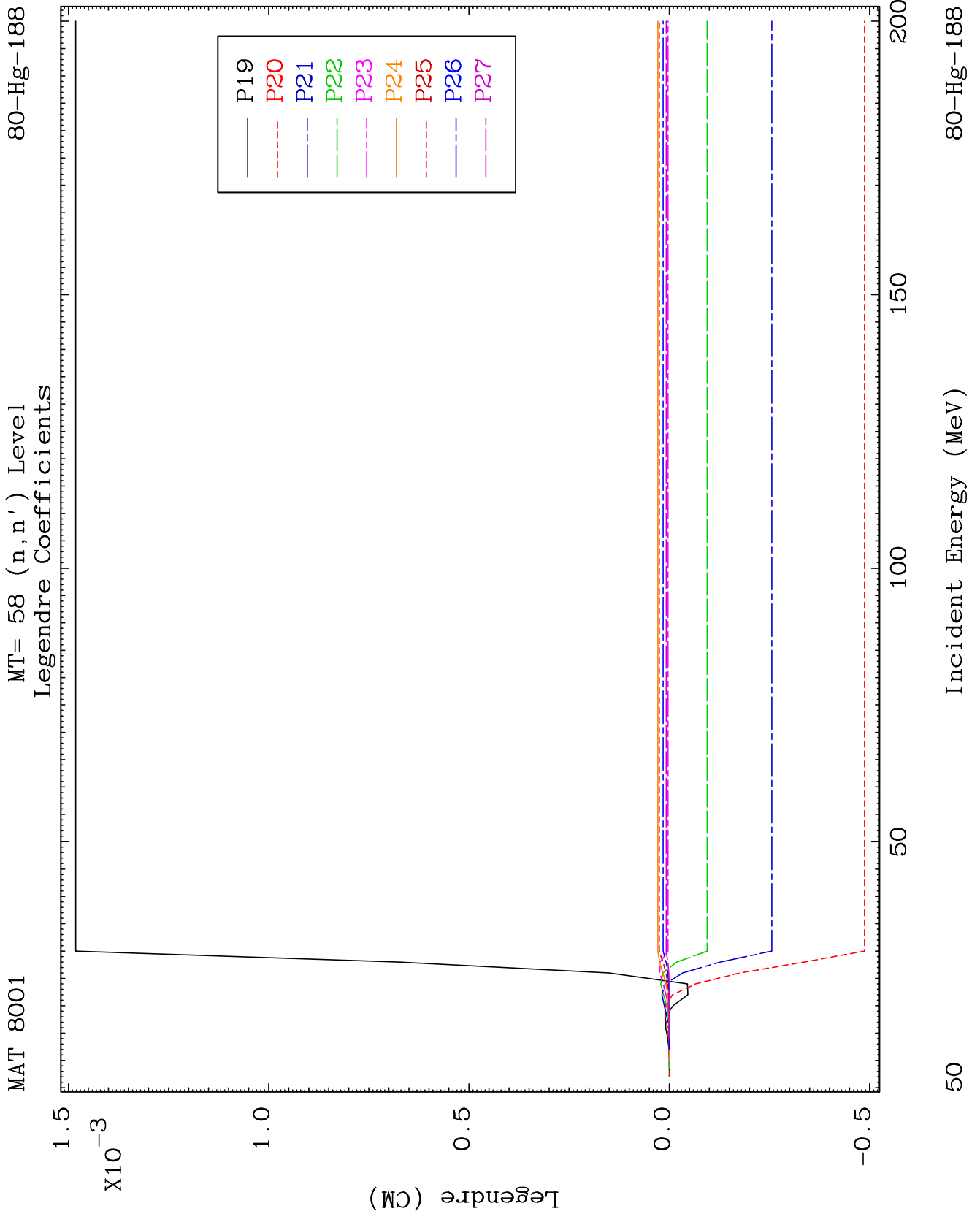


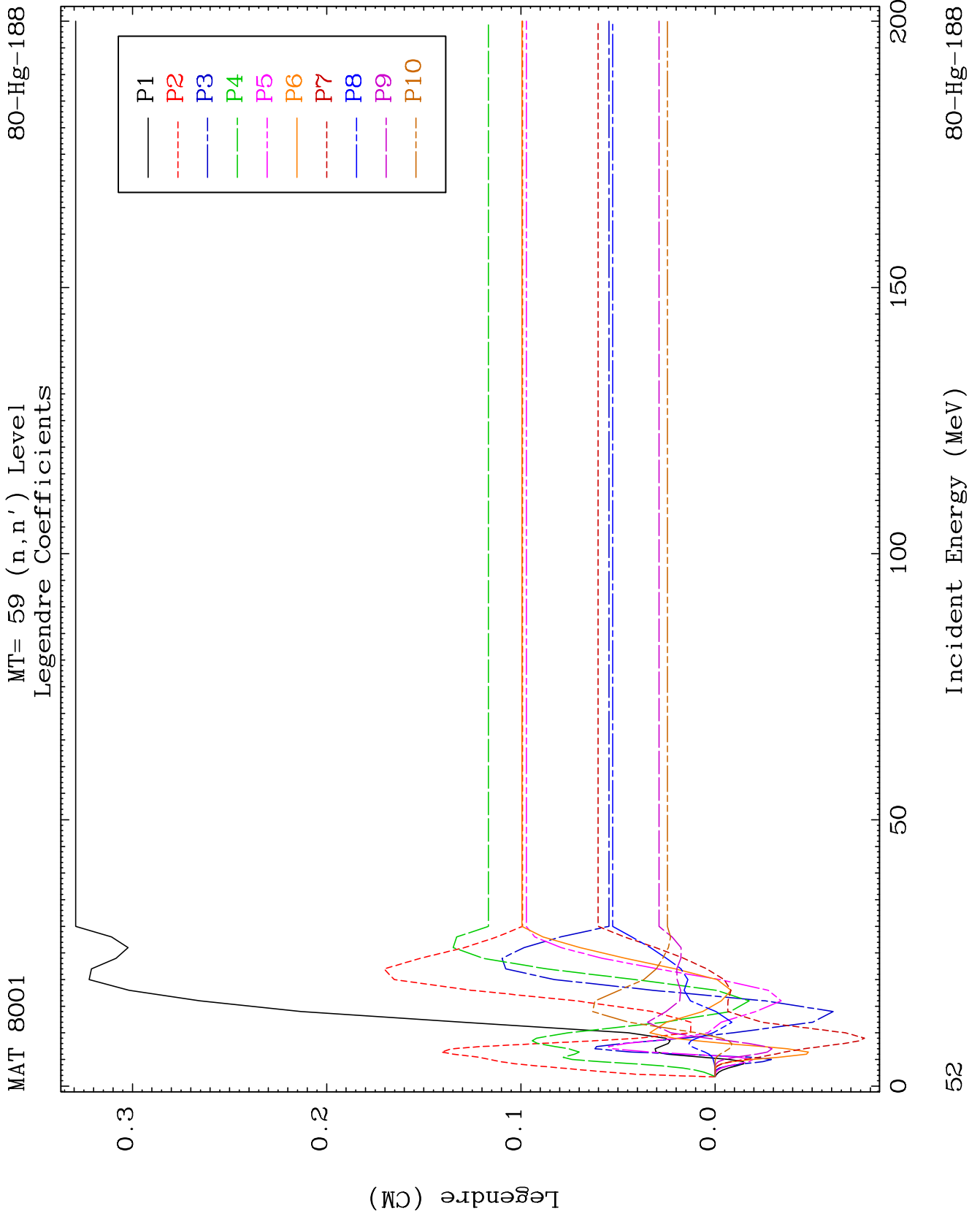


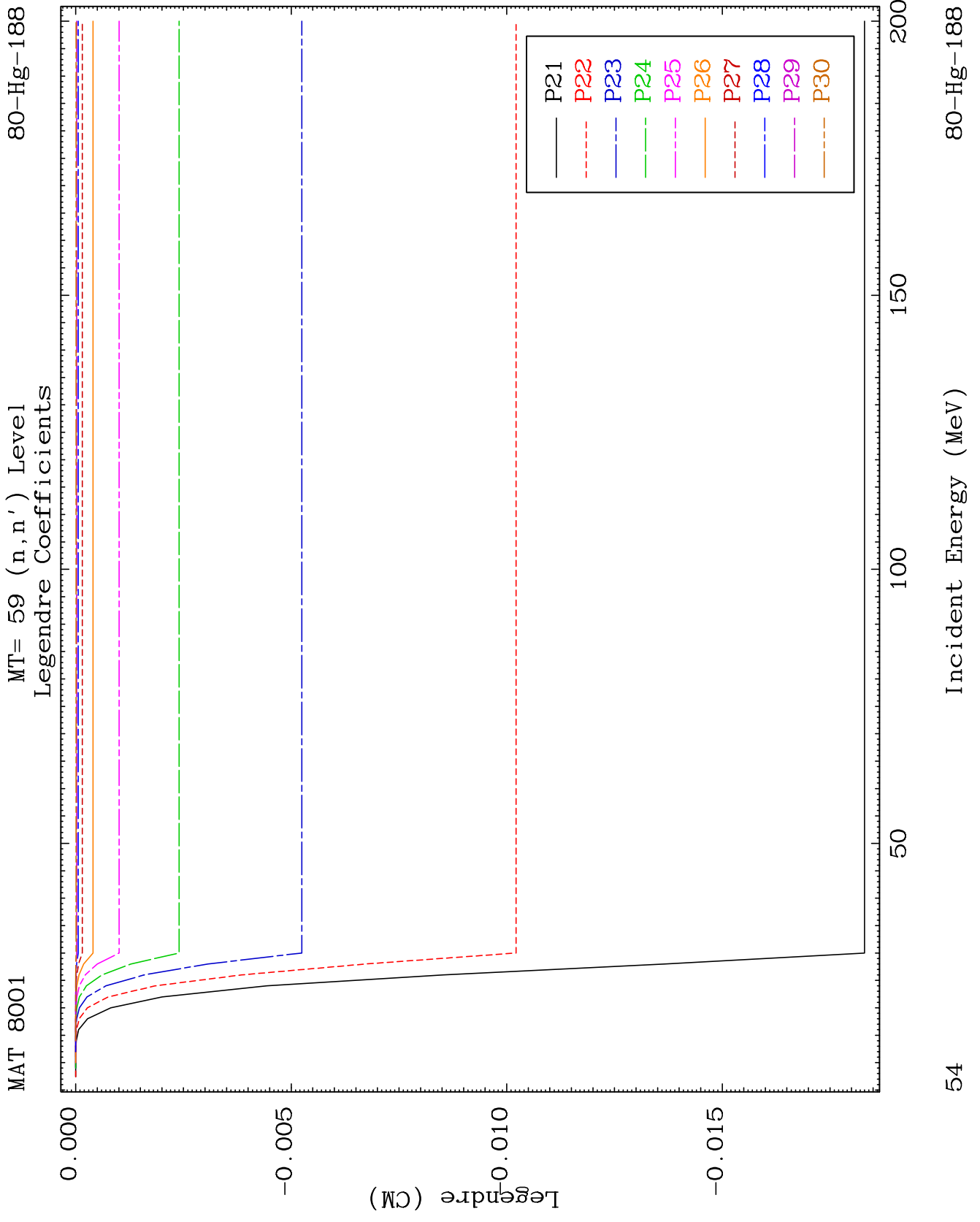


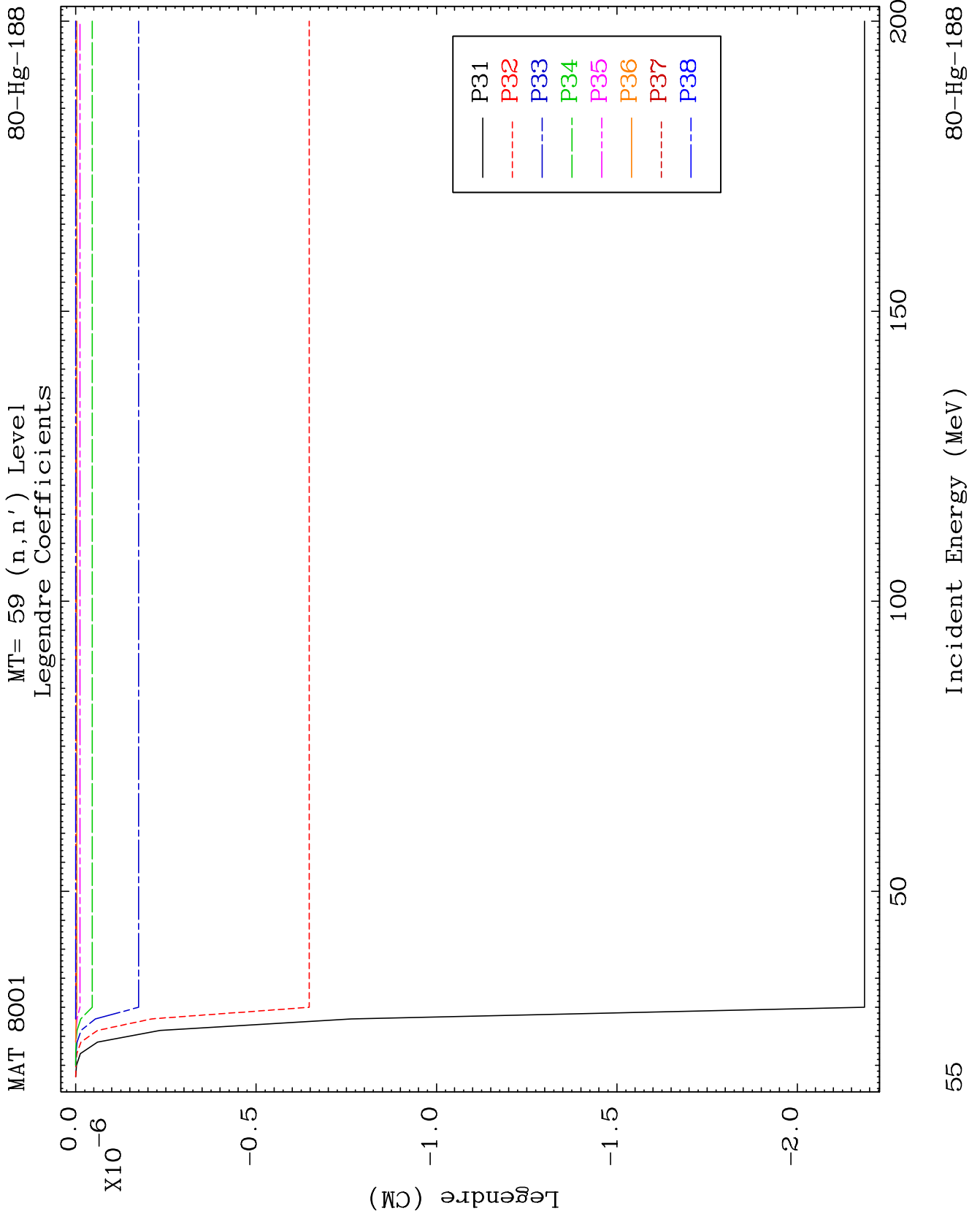


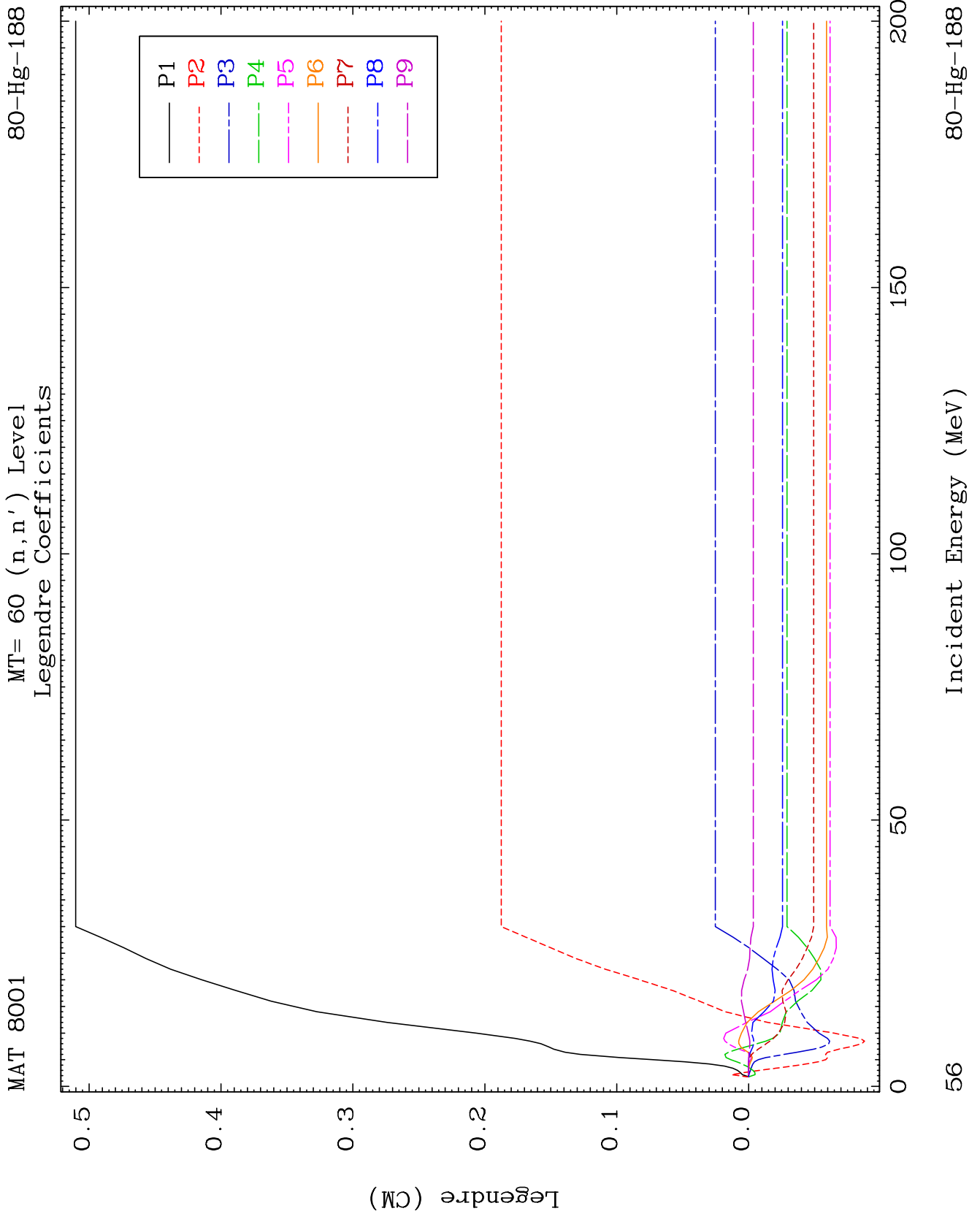








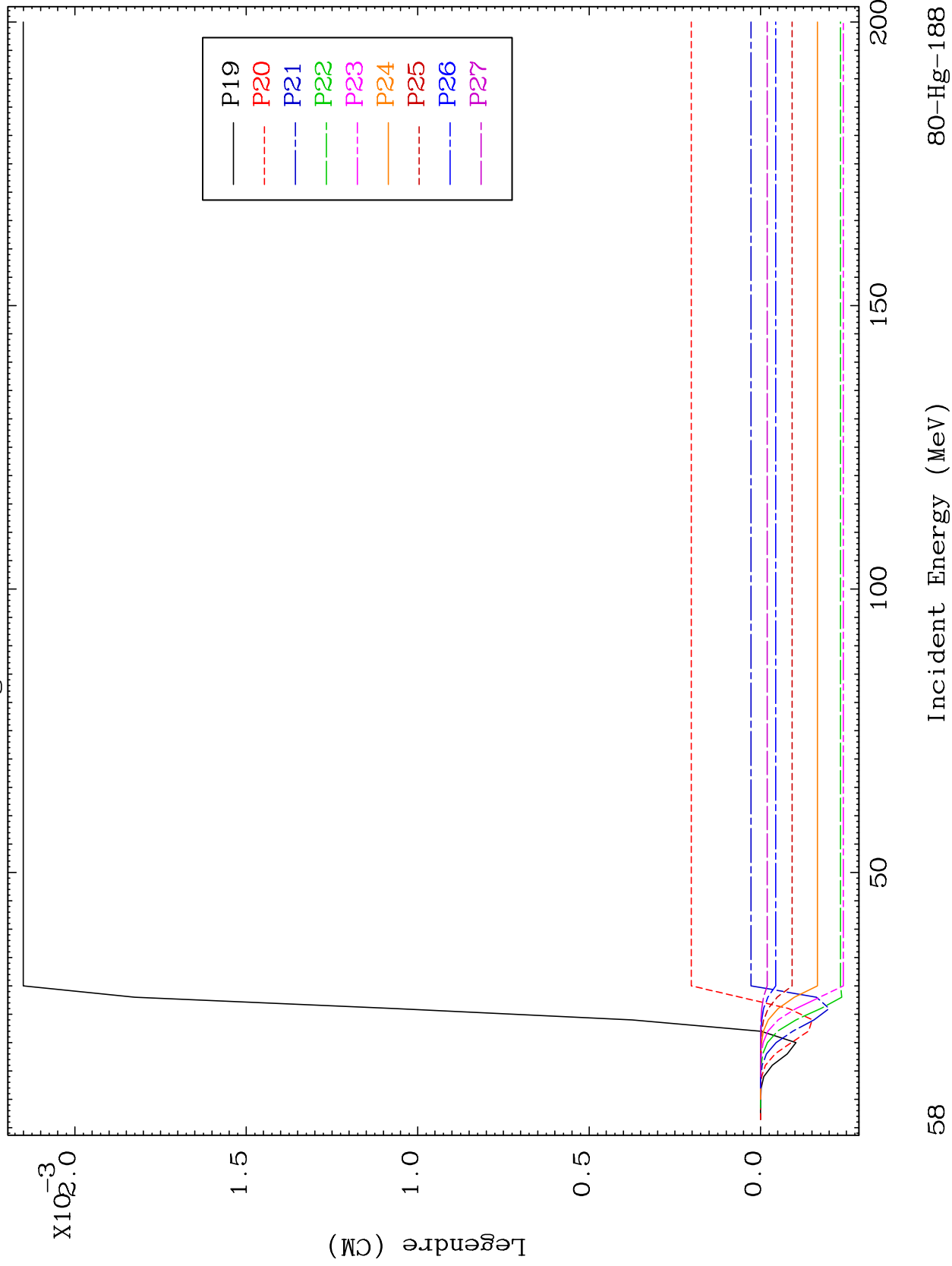




MAT 8001

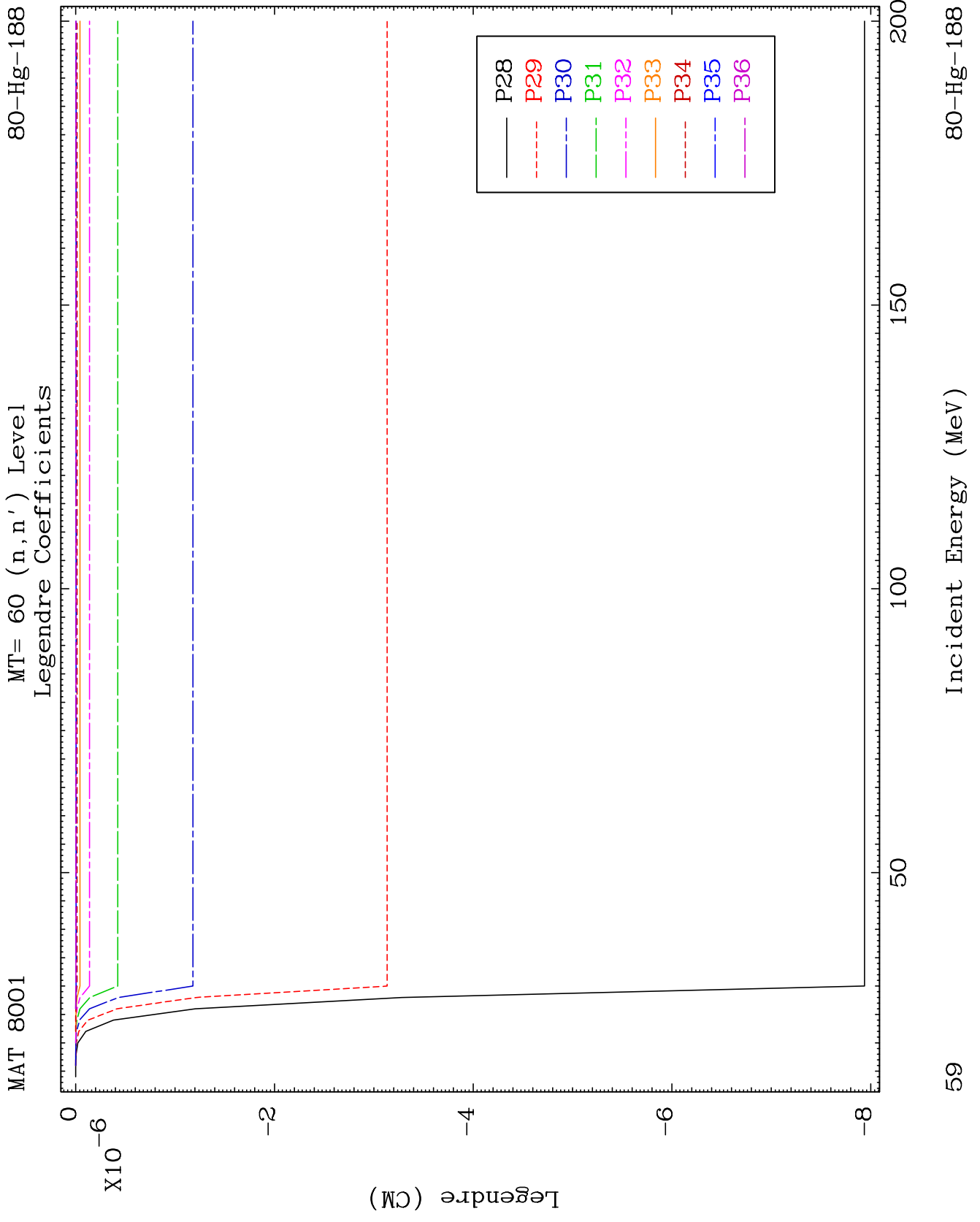
MT= 60 (n,n') Level
Legendre Coefficients

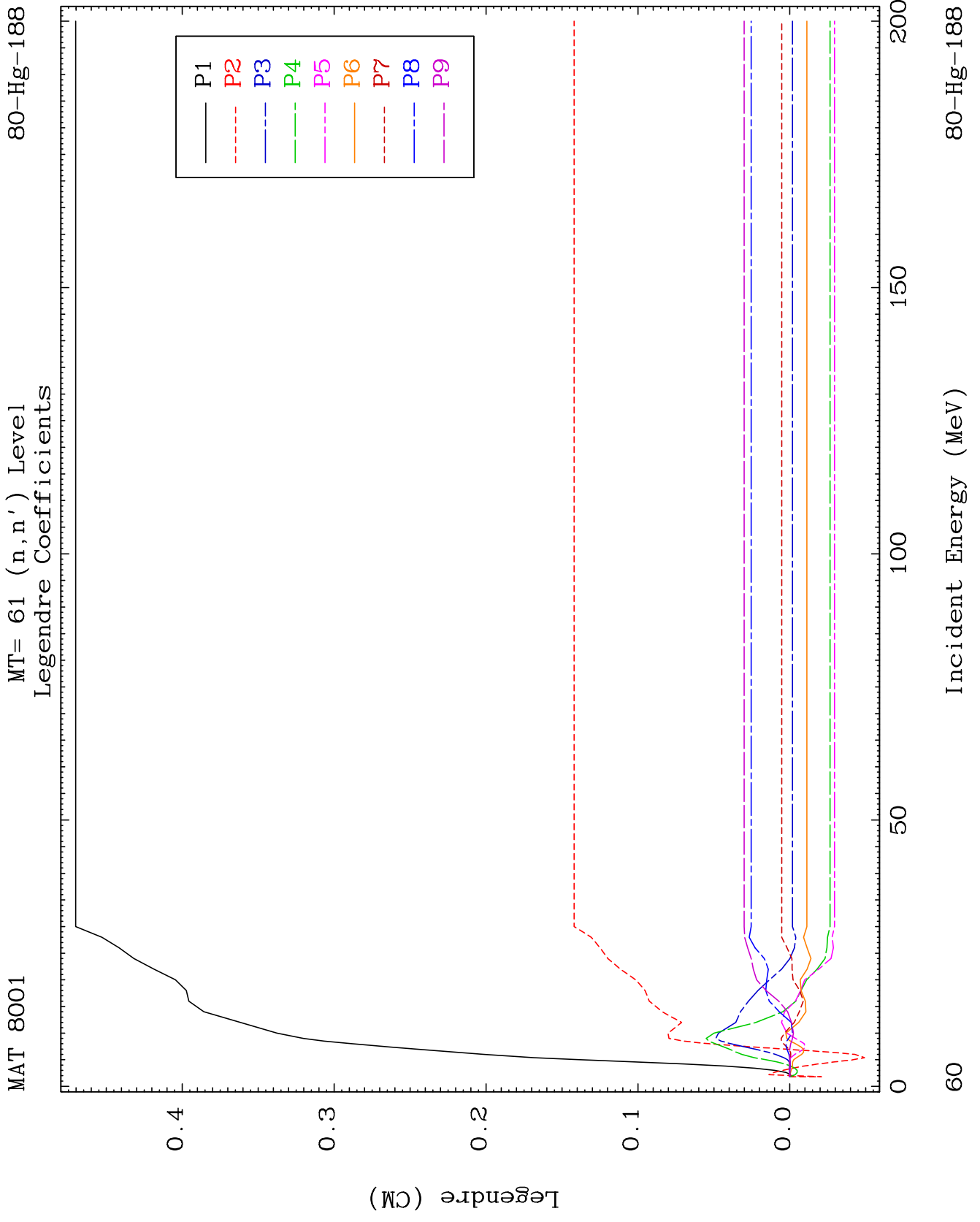
80-Hg-188

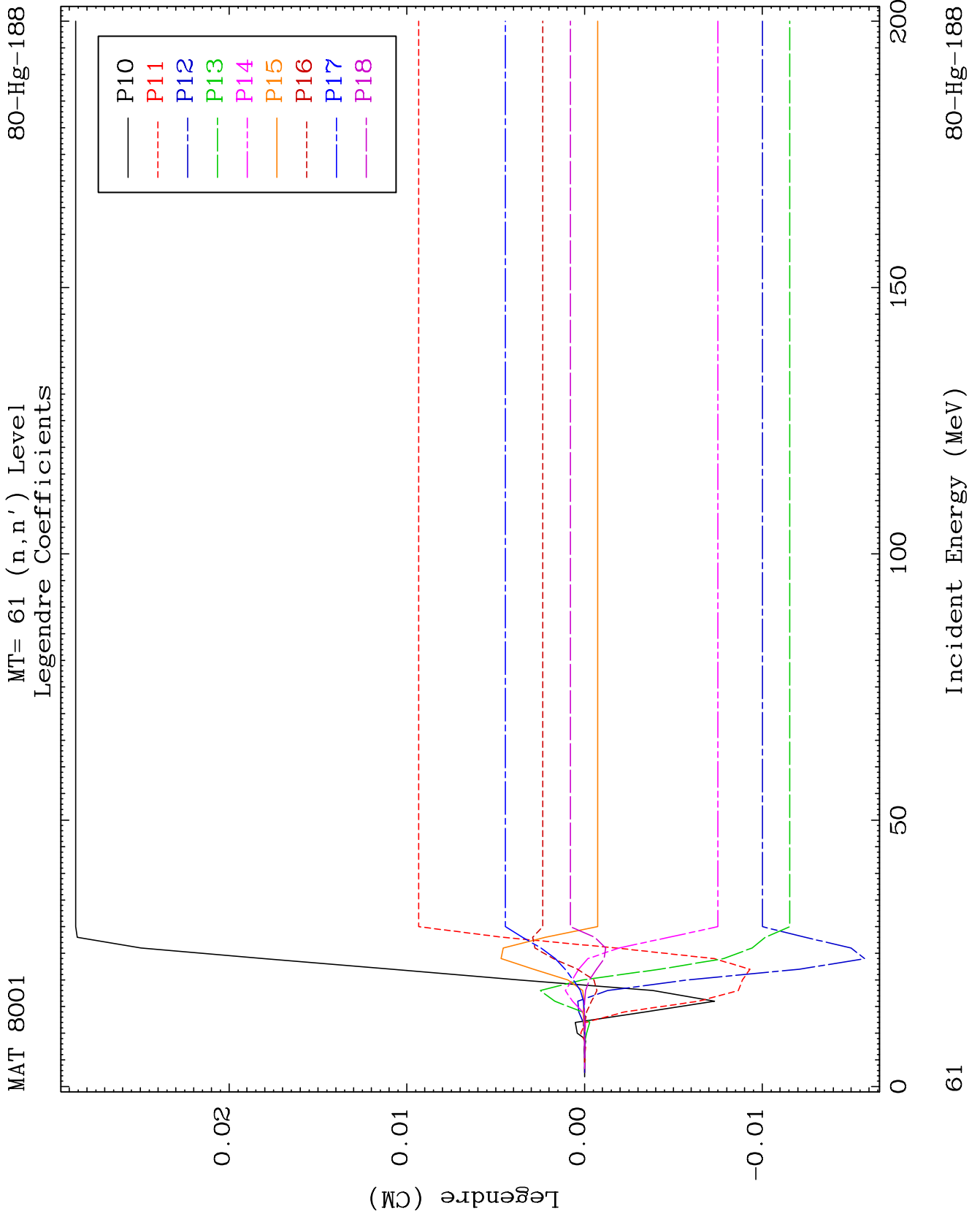


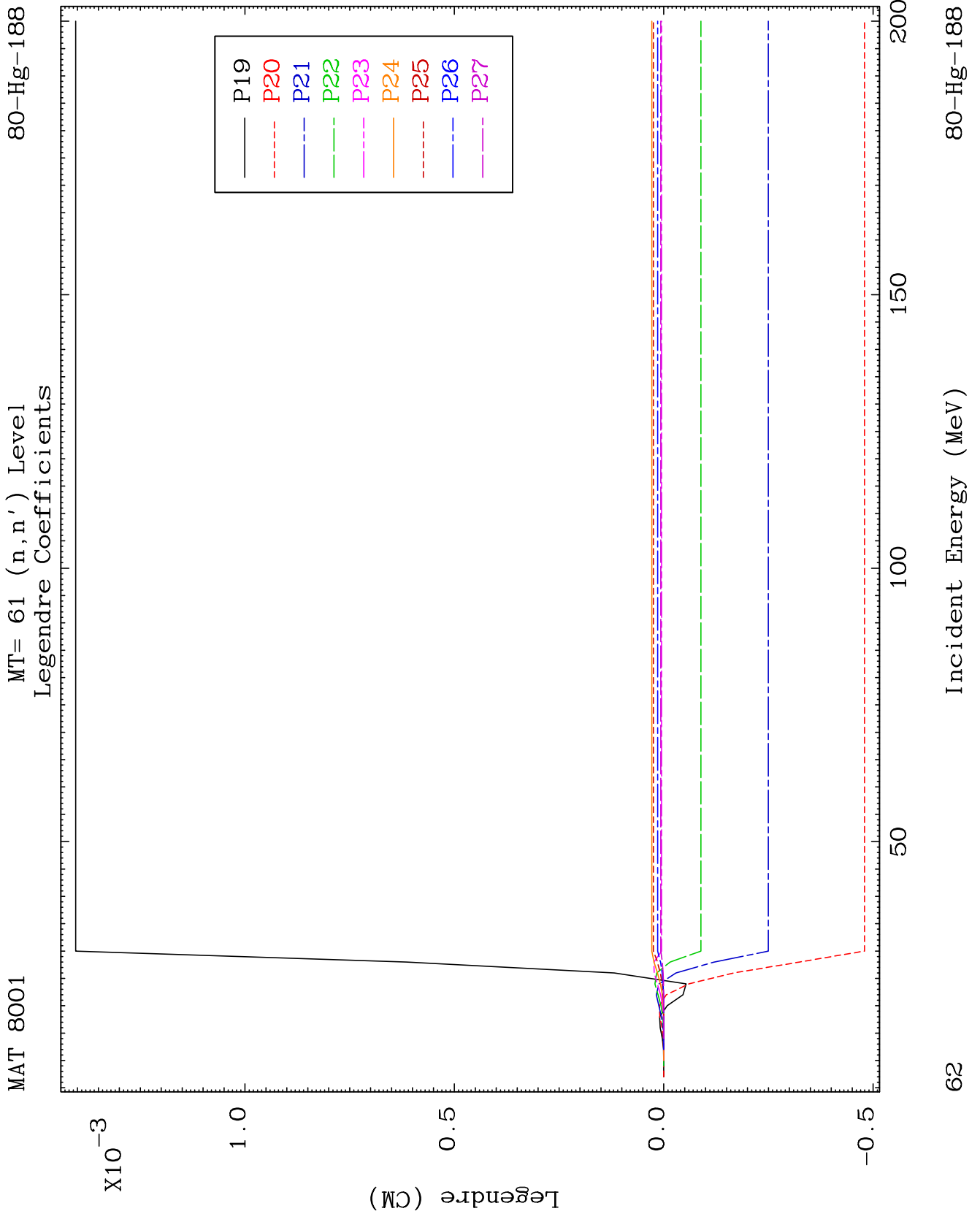
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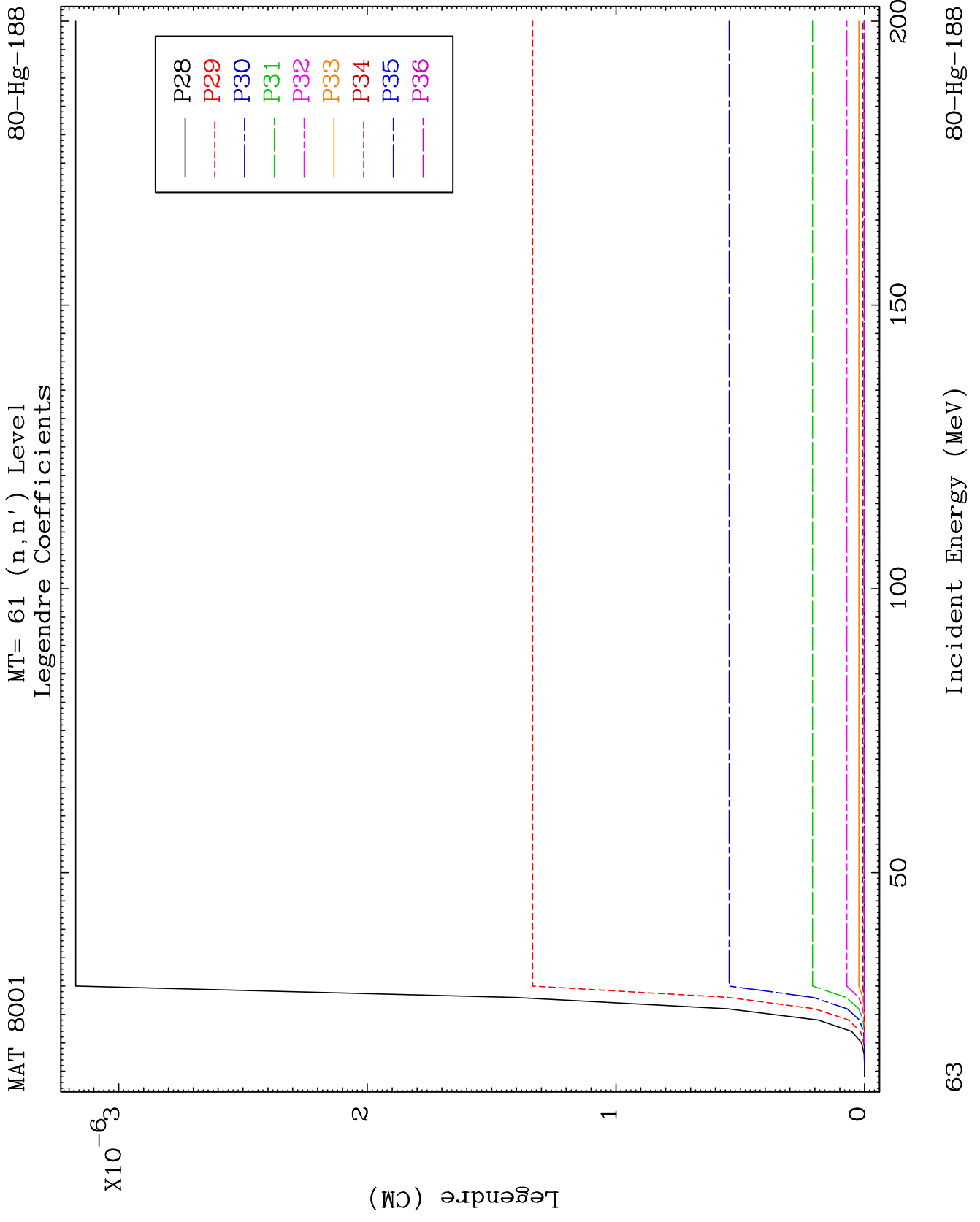
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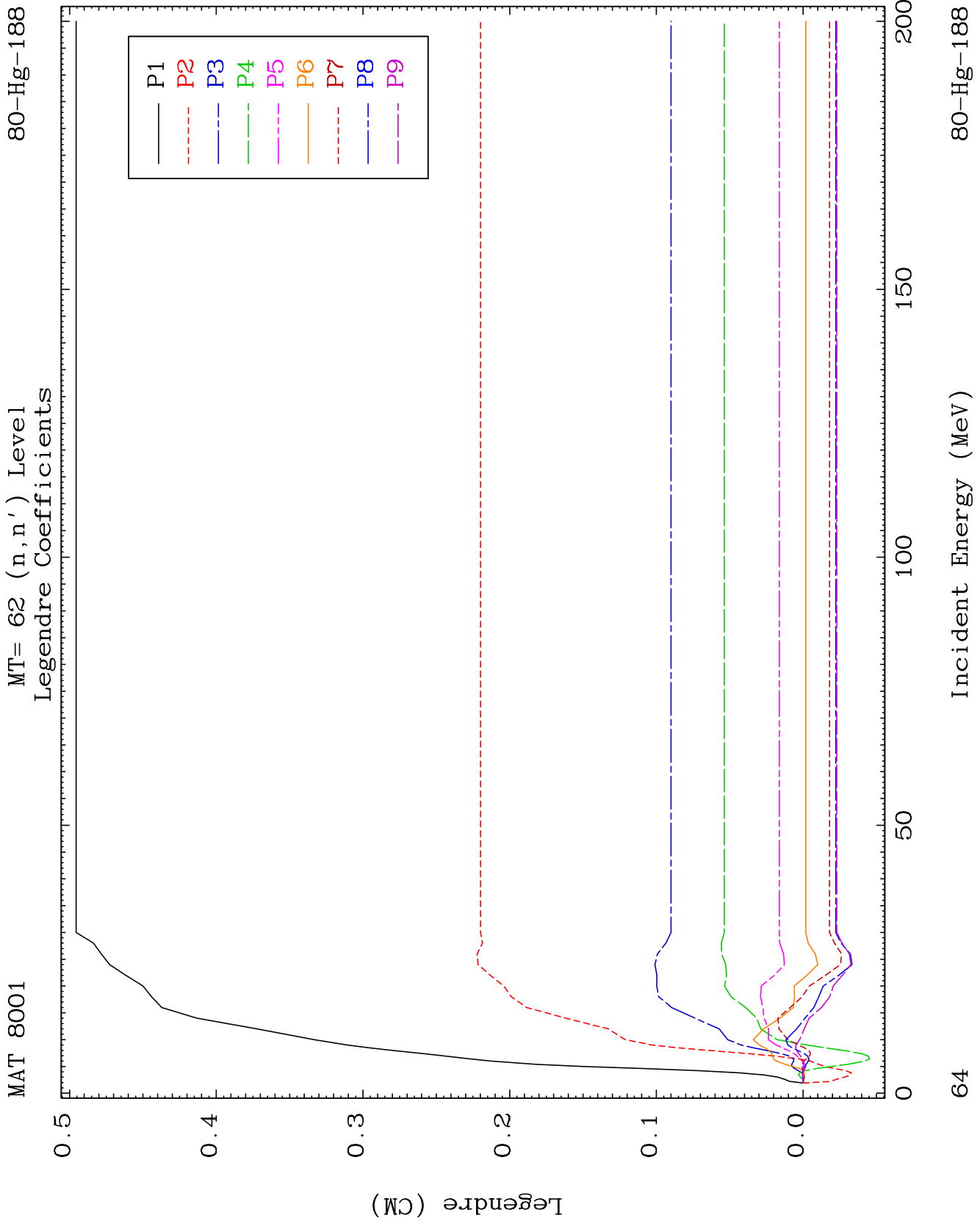


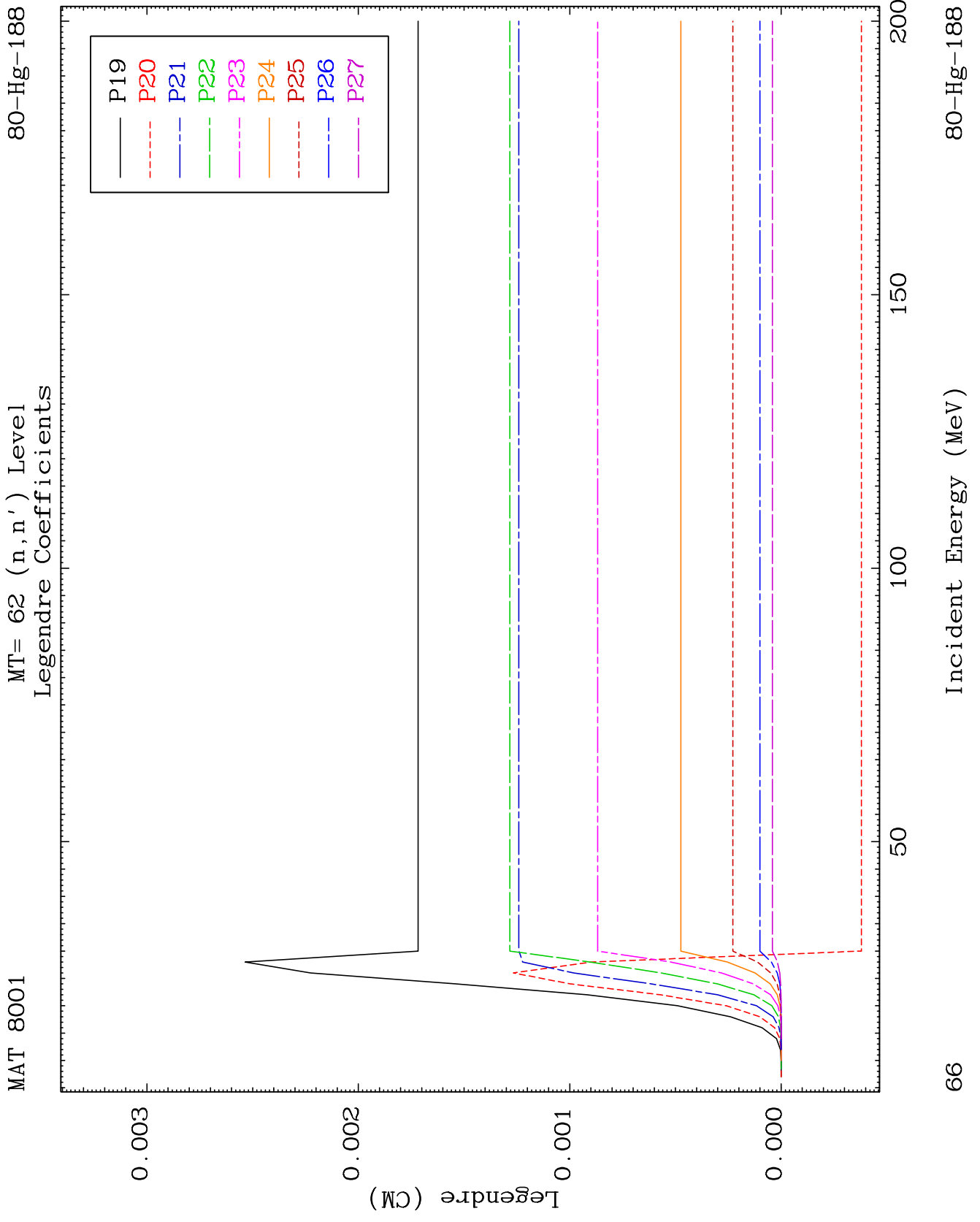


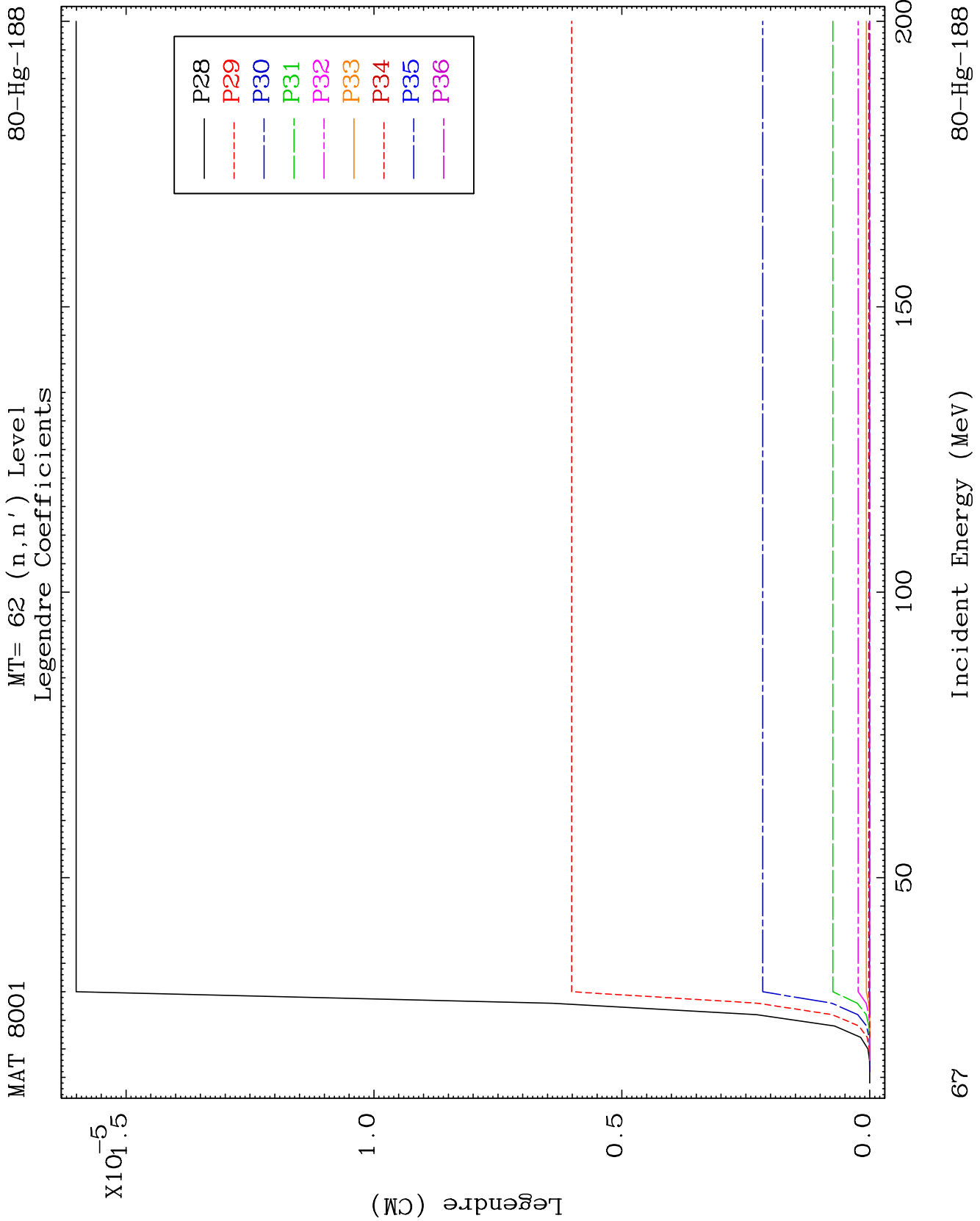


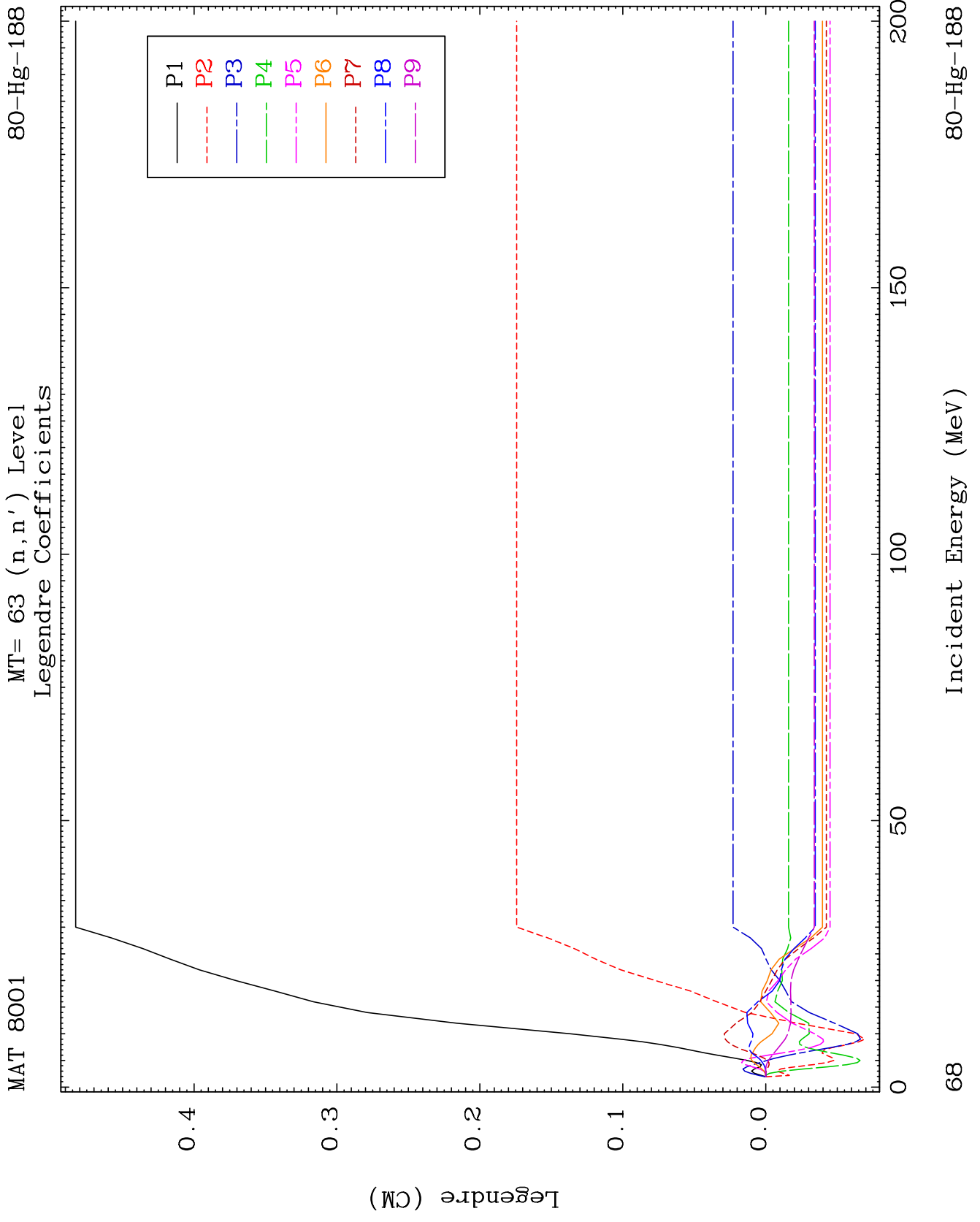








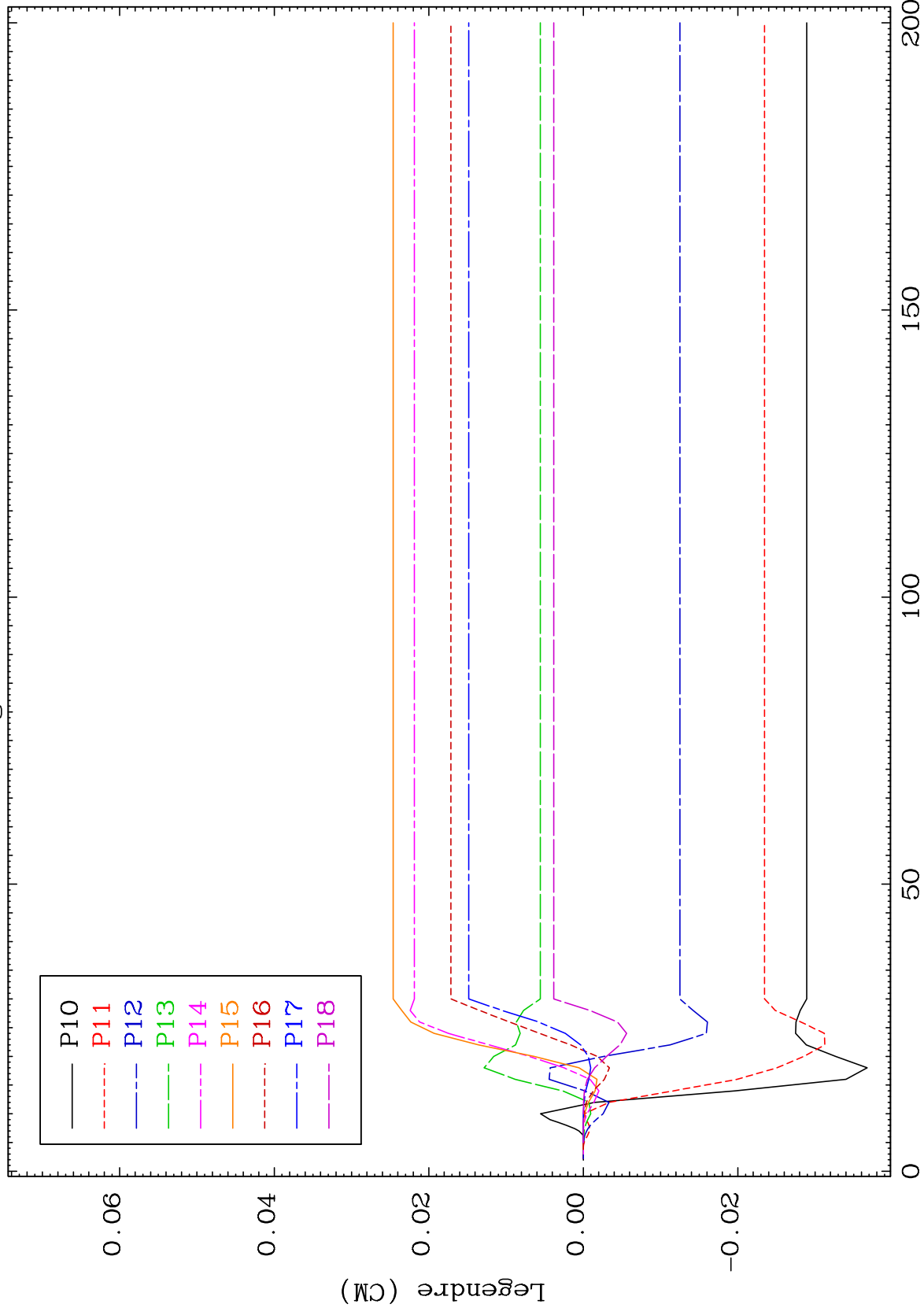




MAT 8001

MT= 63 (n,n') Level
Legendre Coefficients

80-Hg-188



69

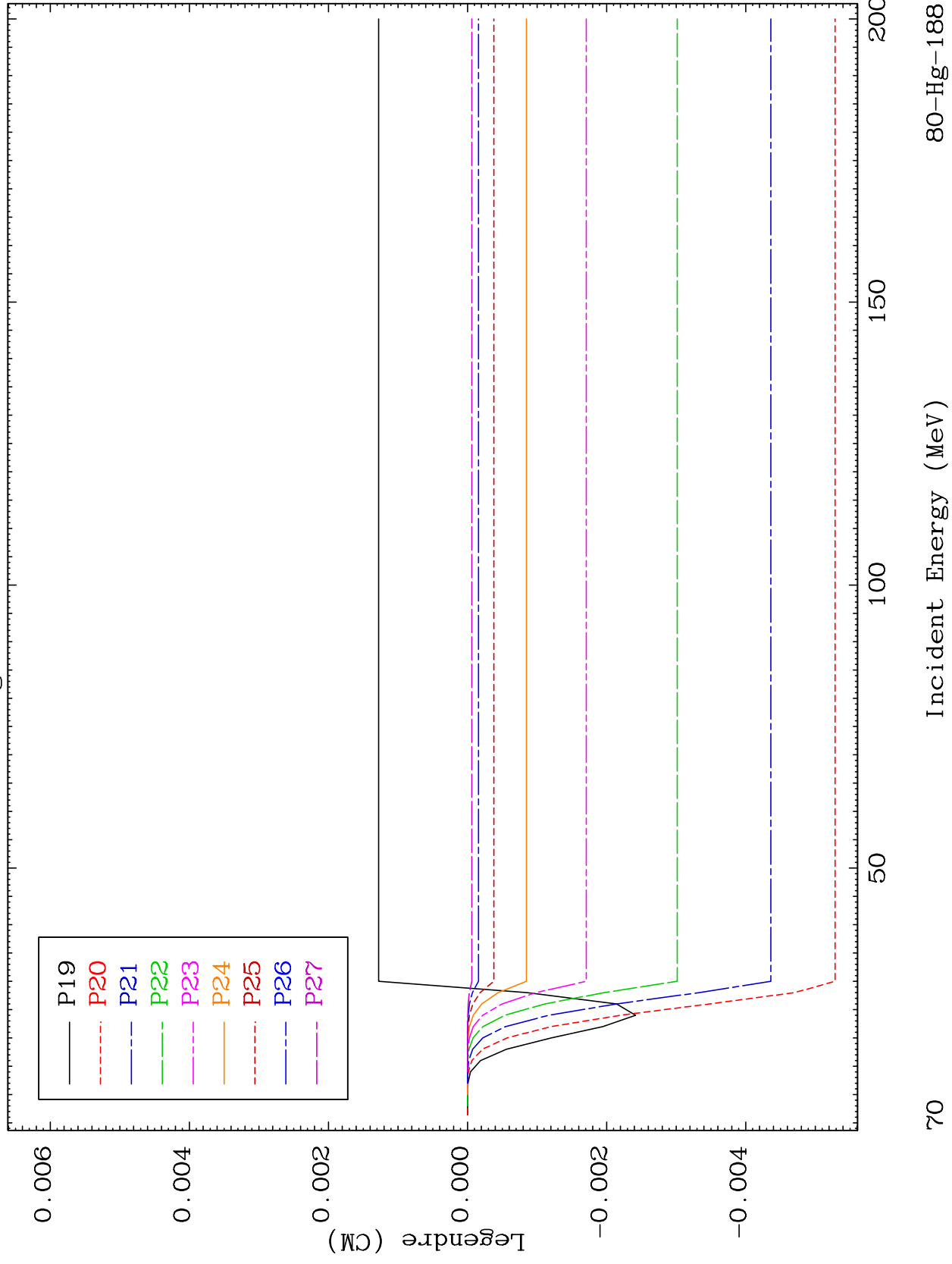
Incident Energy (MeV)

80-Hg-188

MAT 8001

MT= 63 (n,n') Level
Legendre Coefficients

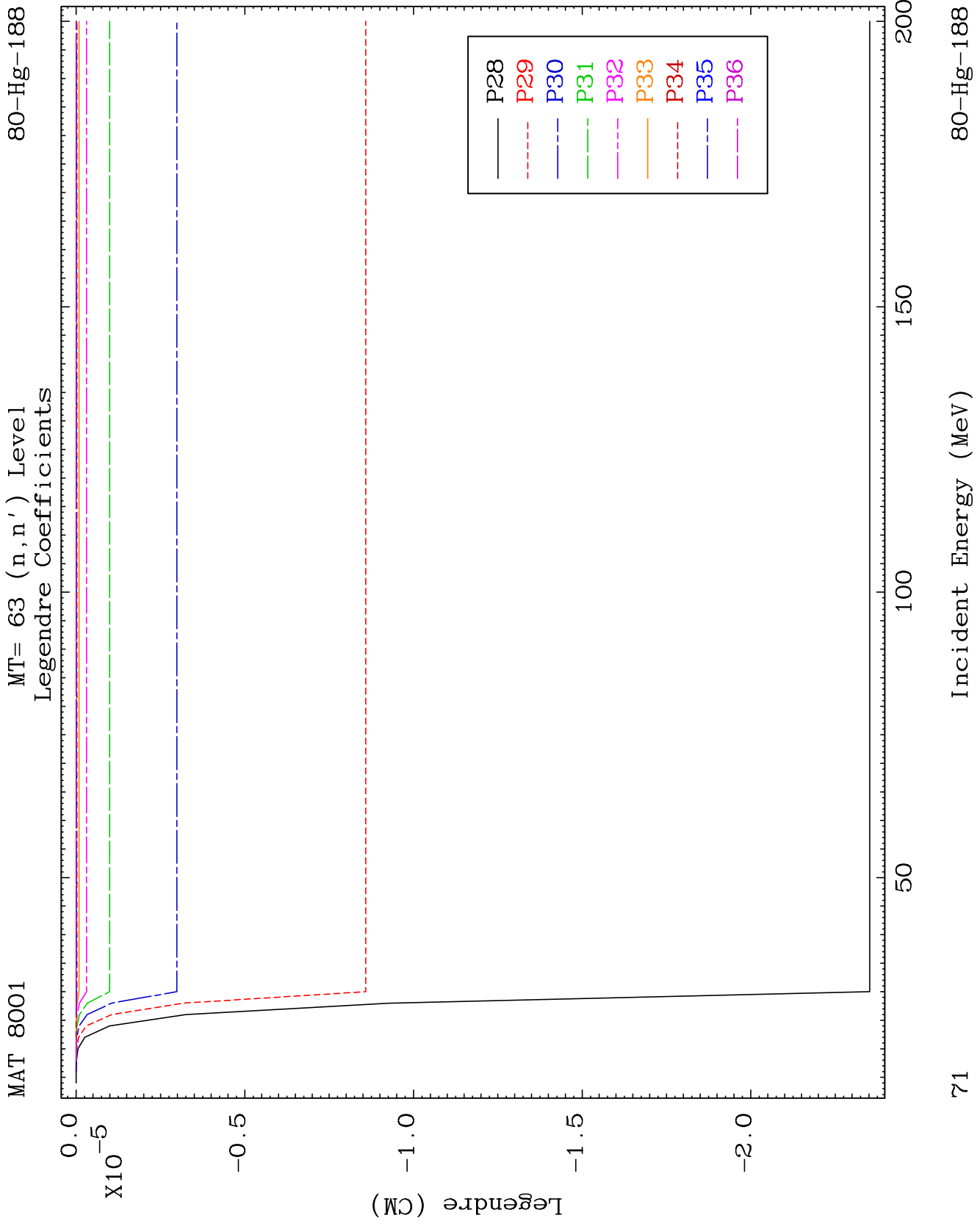
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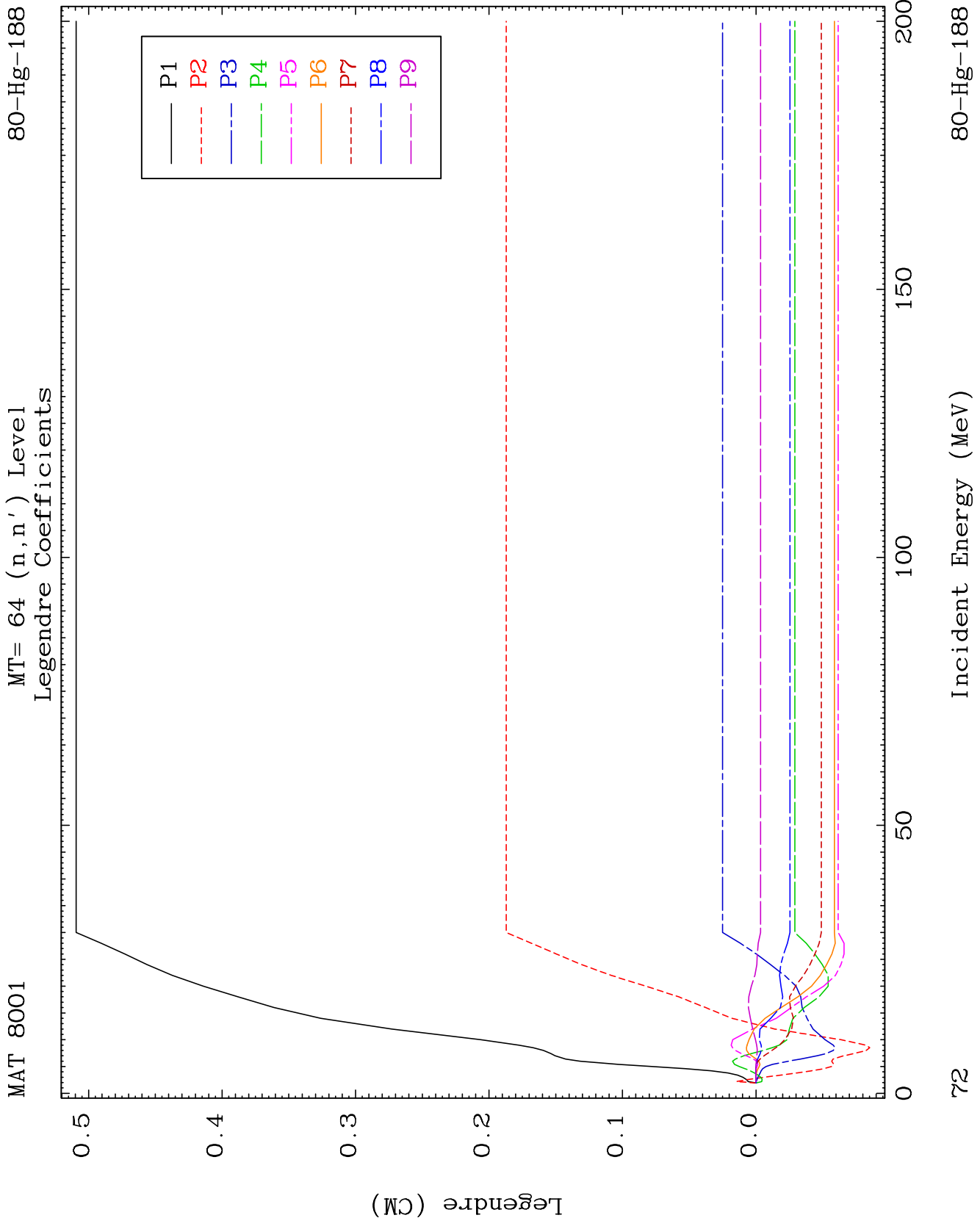


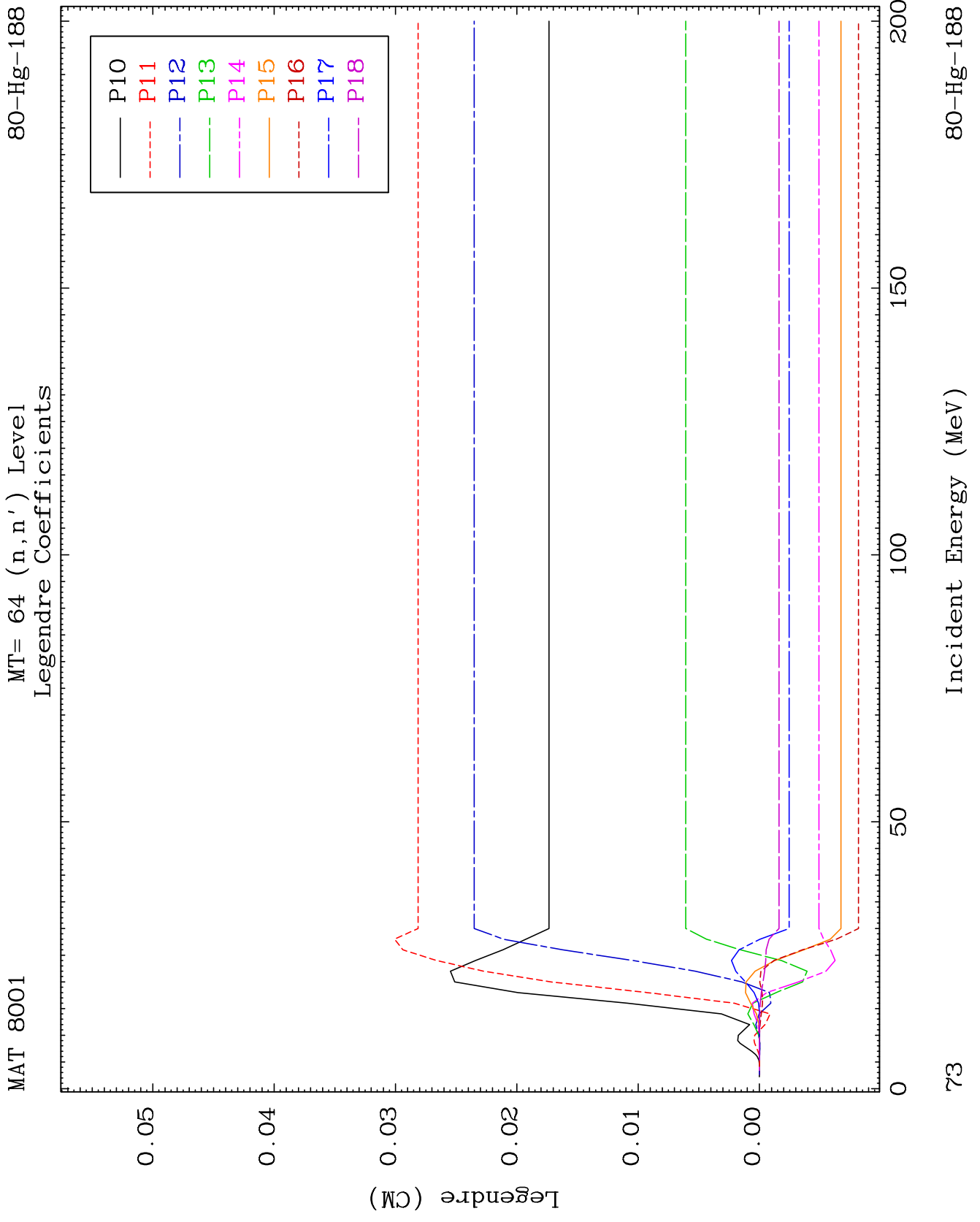
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Incident Energy (MeV)

80-Hg-188



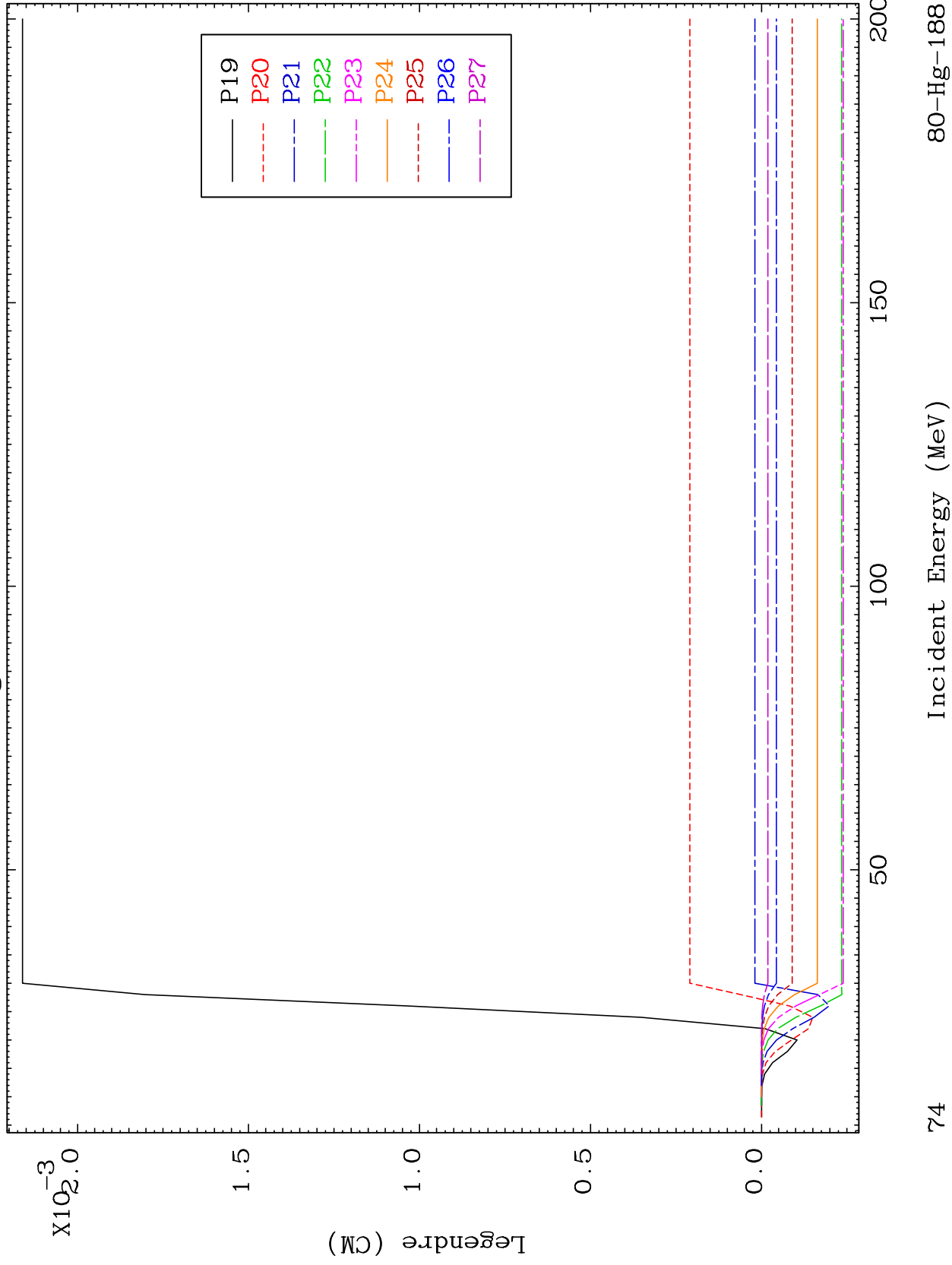




MAT 8001

MT= 64 (n,n') Level
Legendre Coefficients

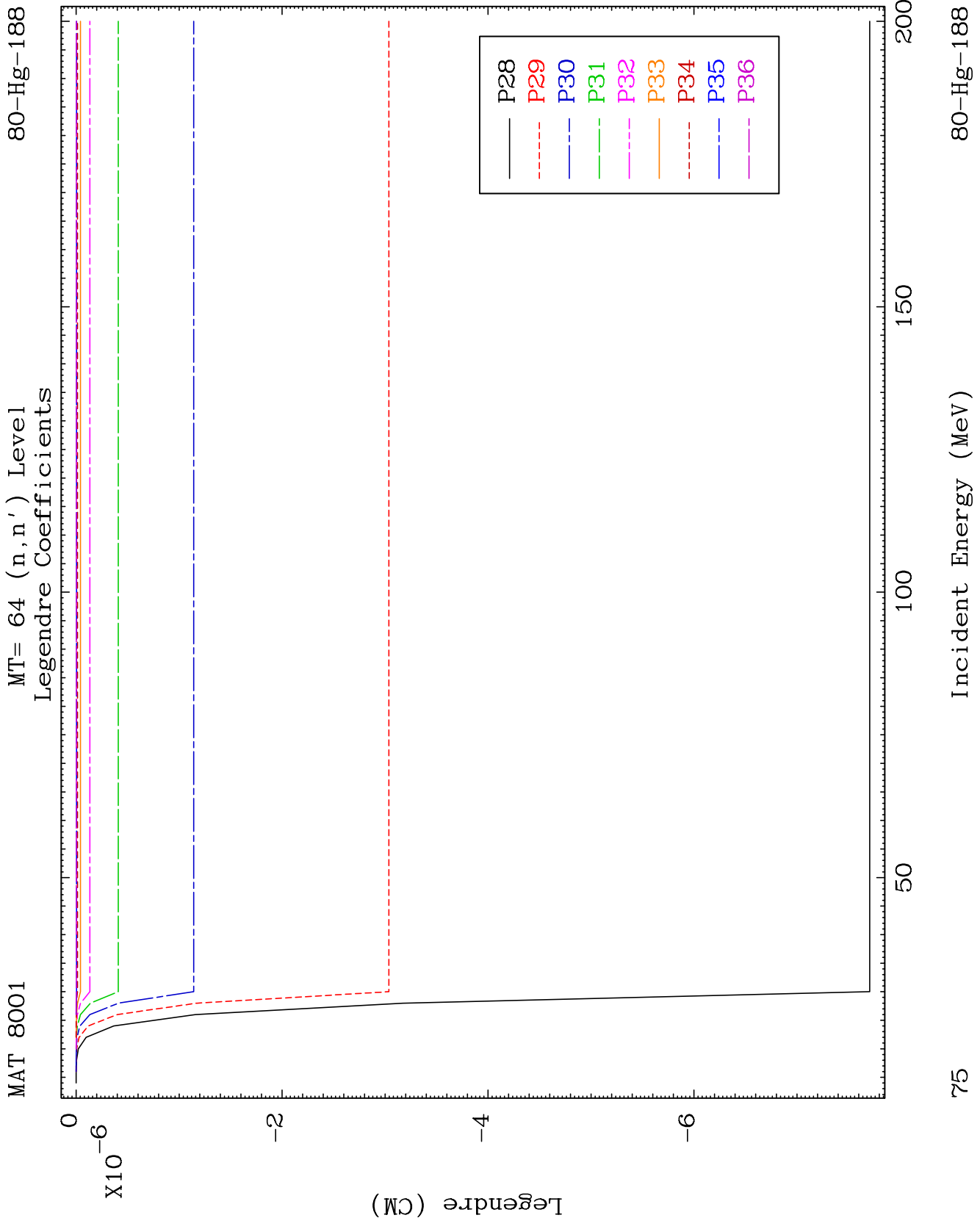
80-Hg-188



74

Incident Energy (MeV)

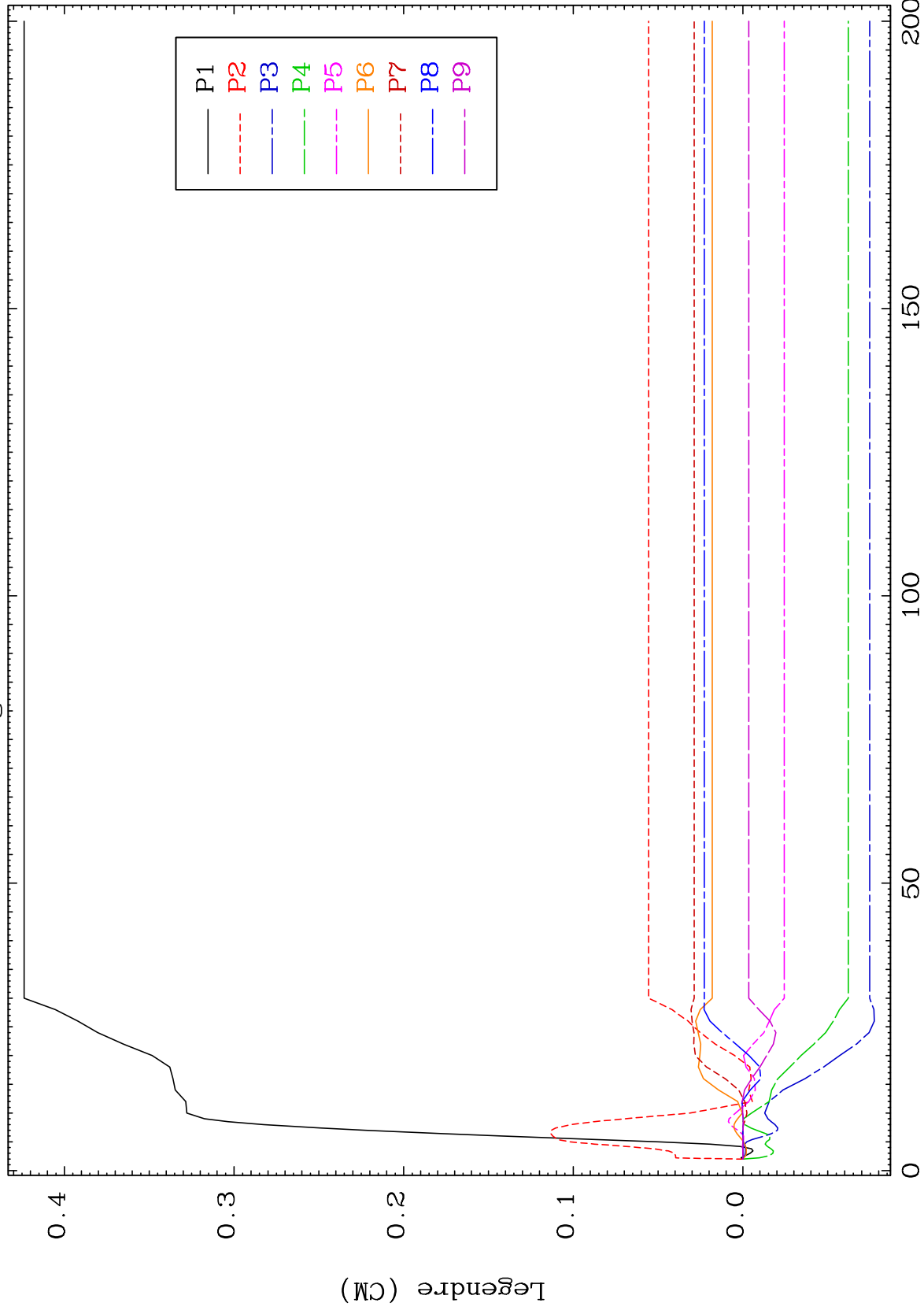
80-Hg-188



MAT 8001

MT= 65 (n,n') Level
Legendre Coefficients

80-Hg-188



76

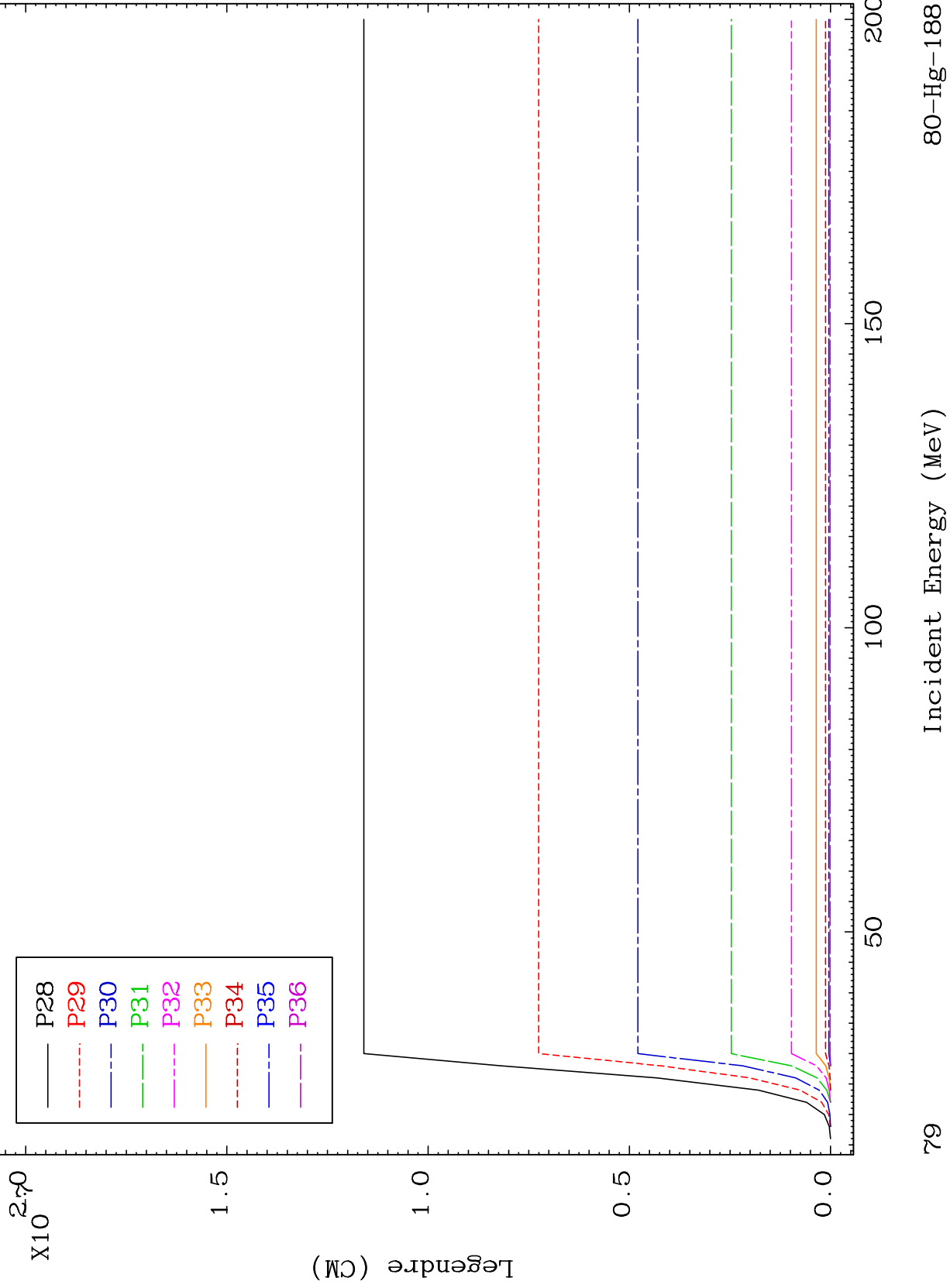
Incident Energy (MeV)

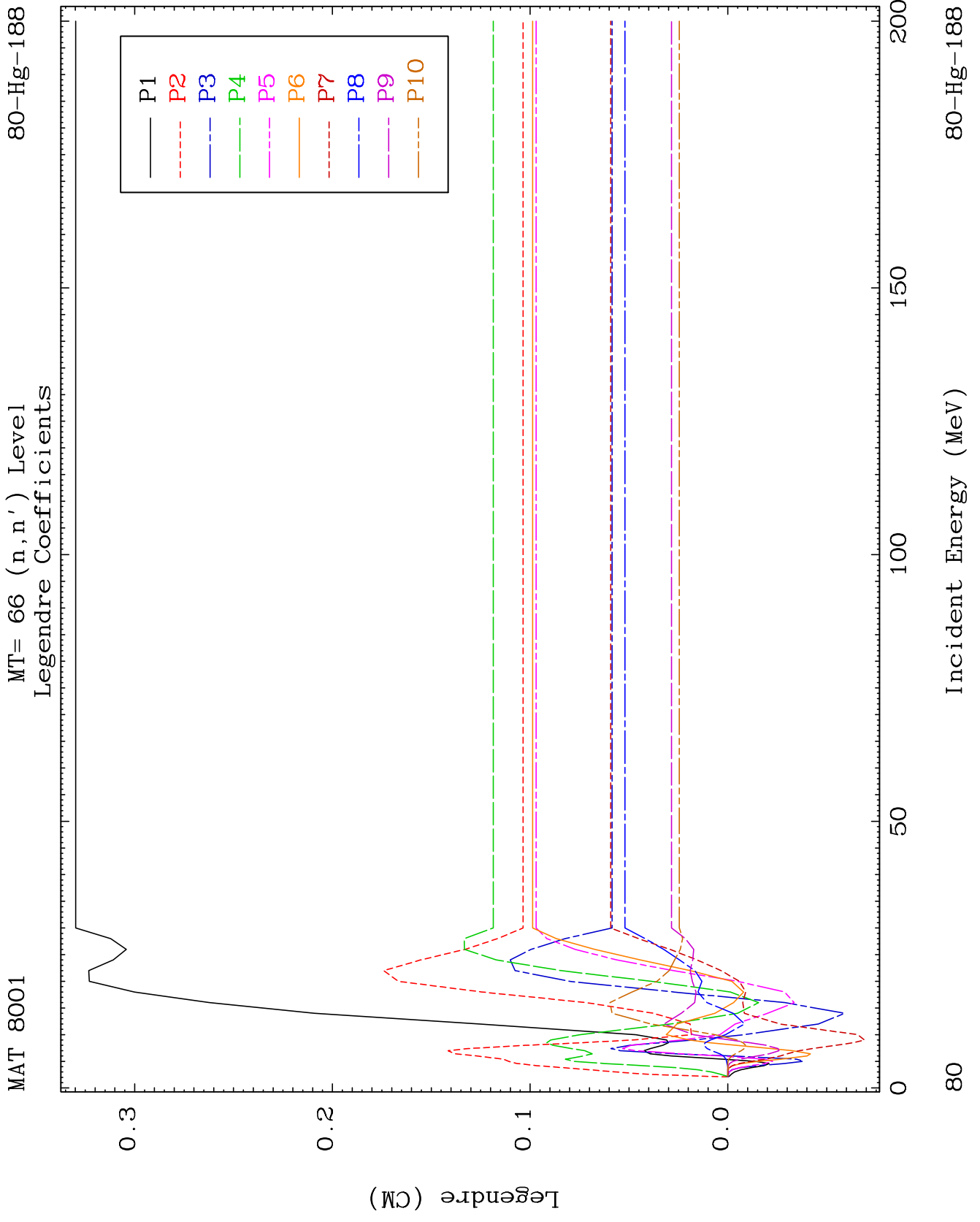
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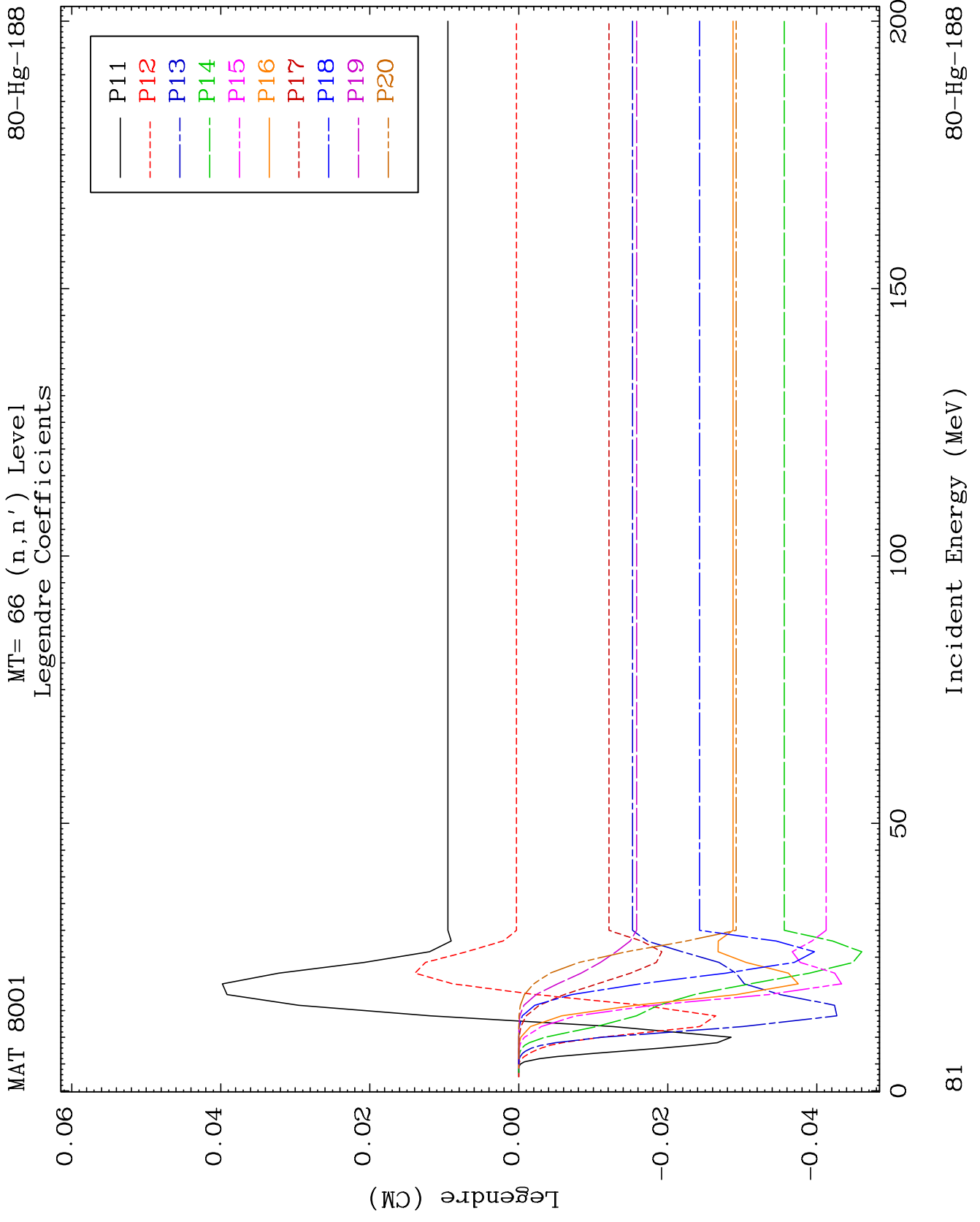
MAT 8001

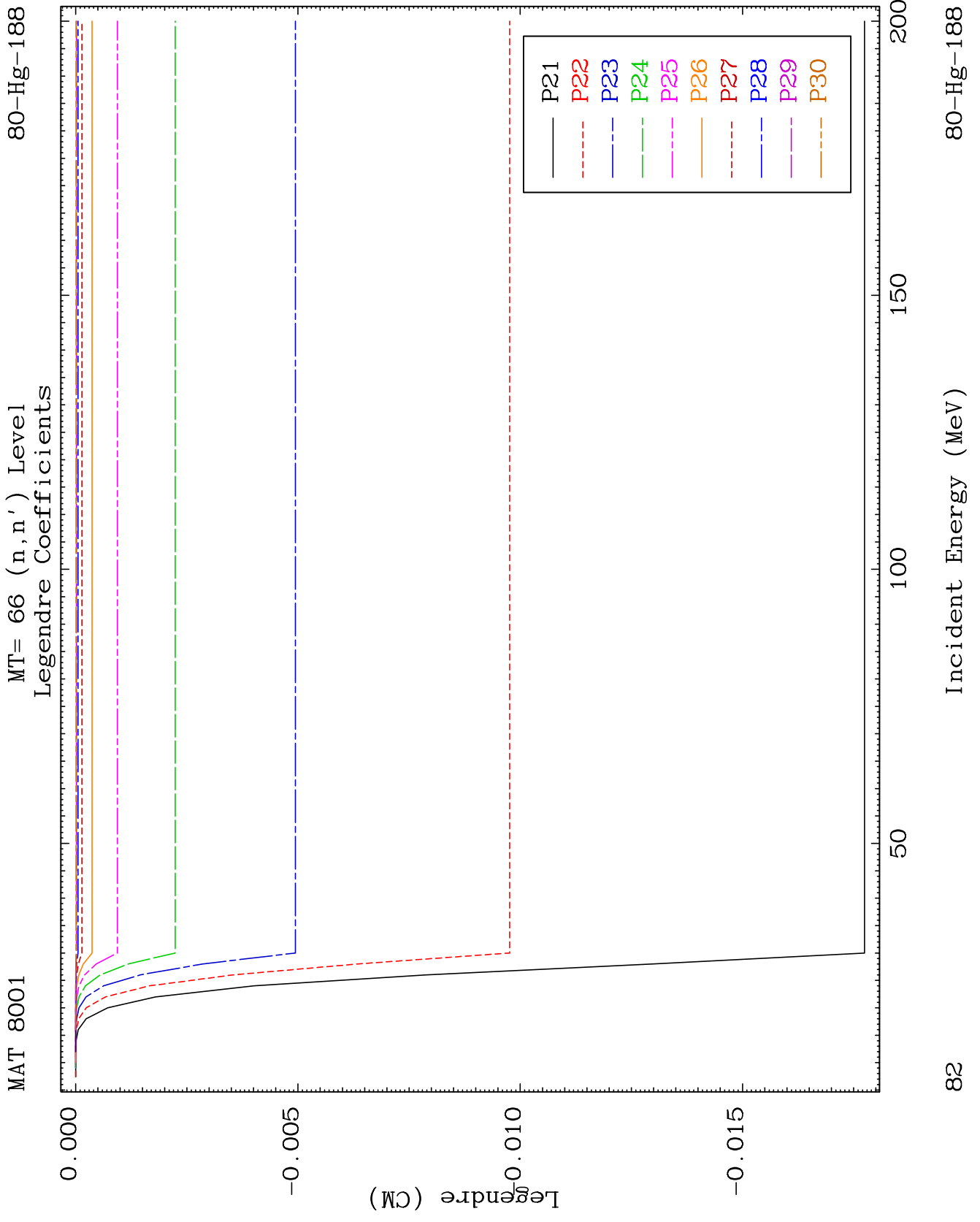
MT= 65 (n,n') Level
Legendre Coefficients

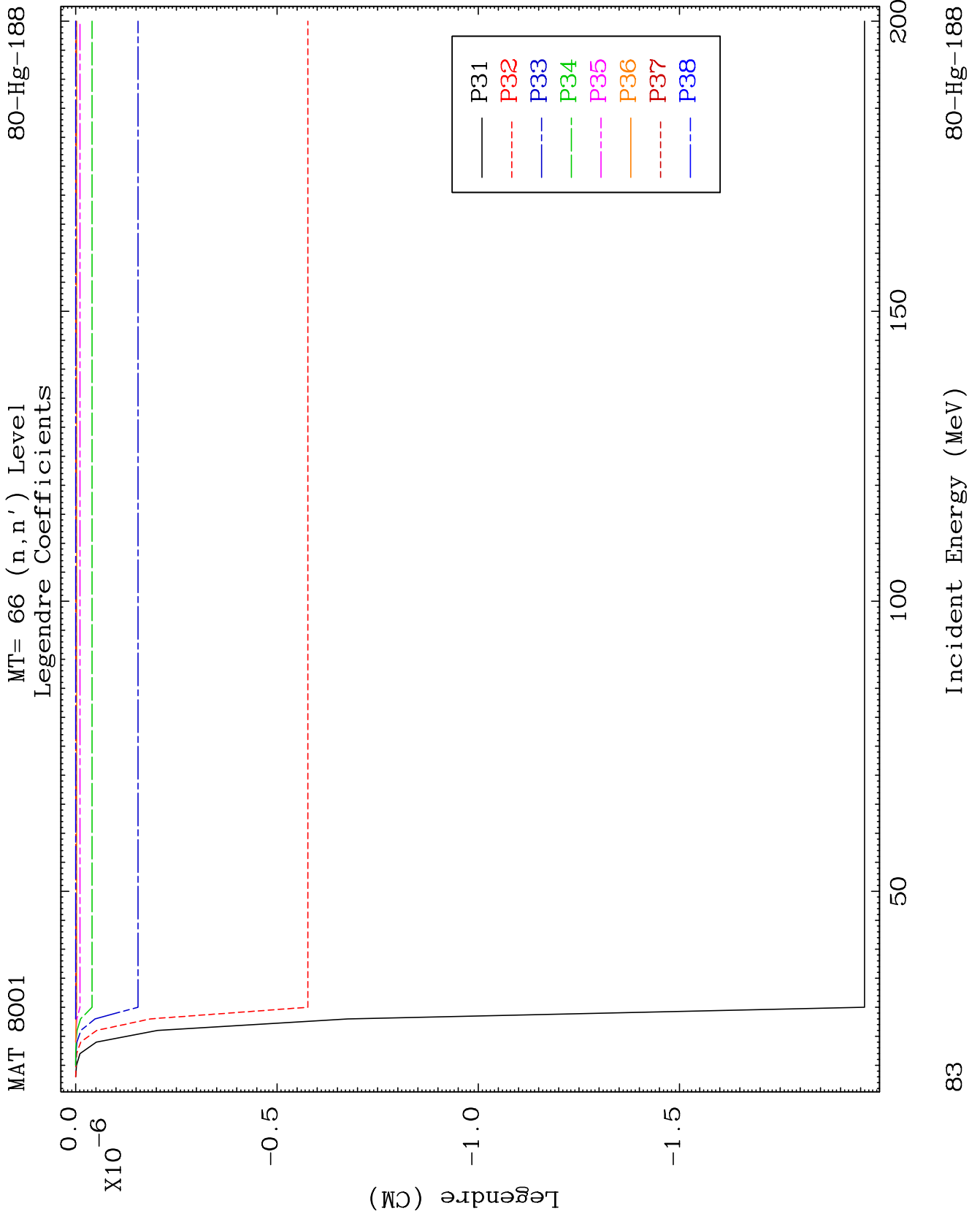
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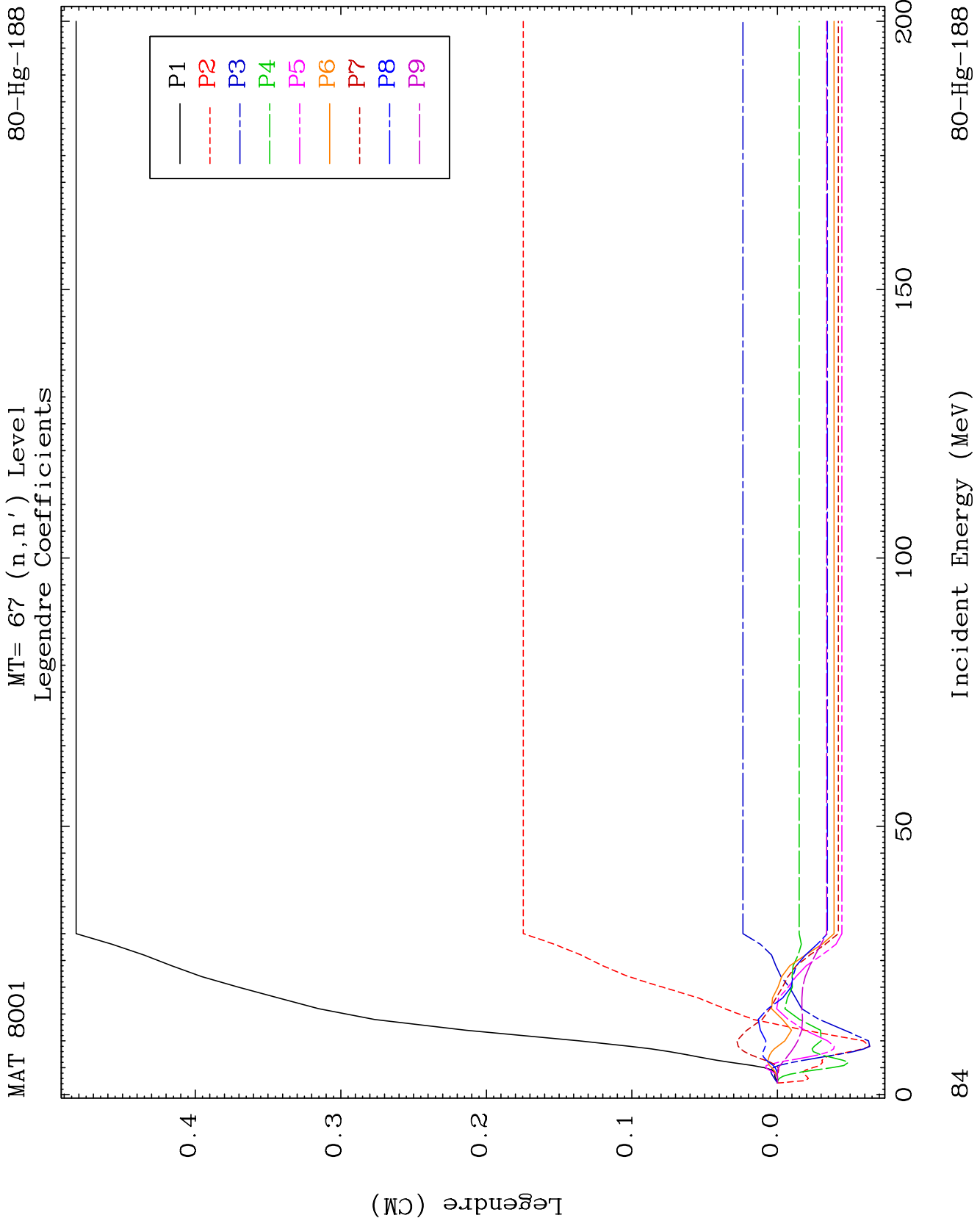


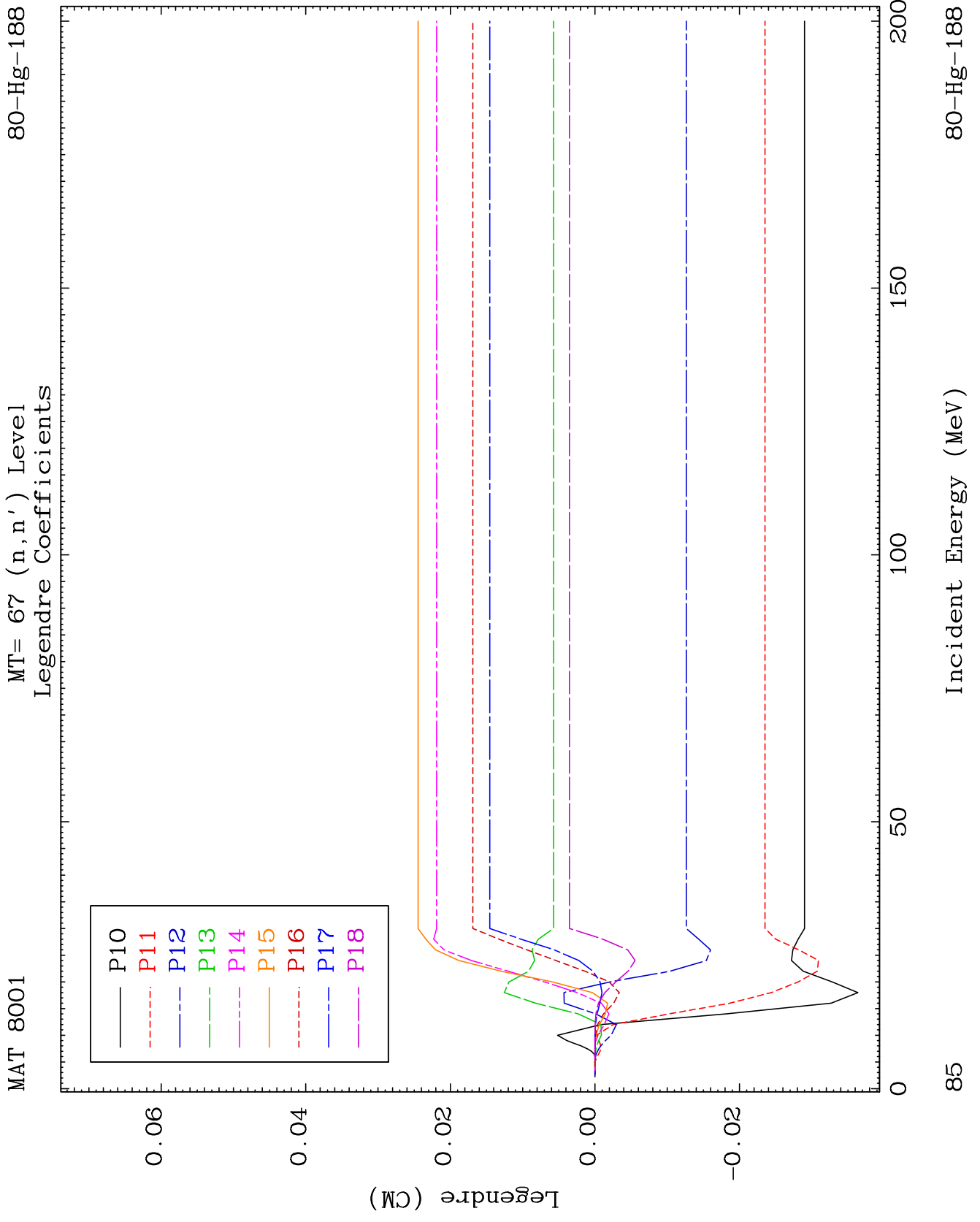








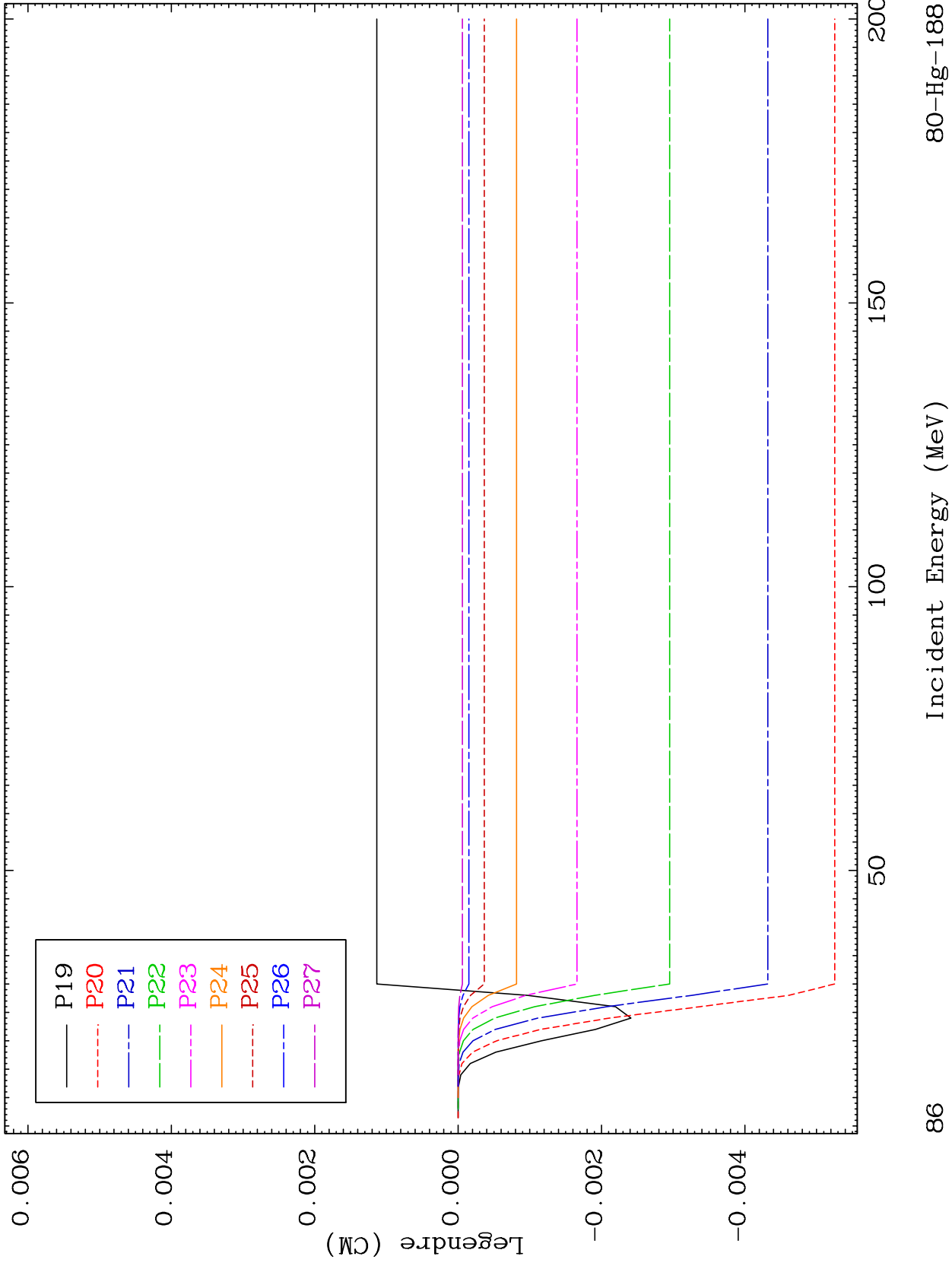




MAT 8001

MT= 67 (n,n') Level
Legendre Coefficients

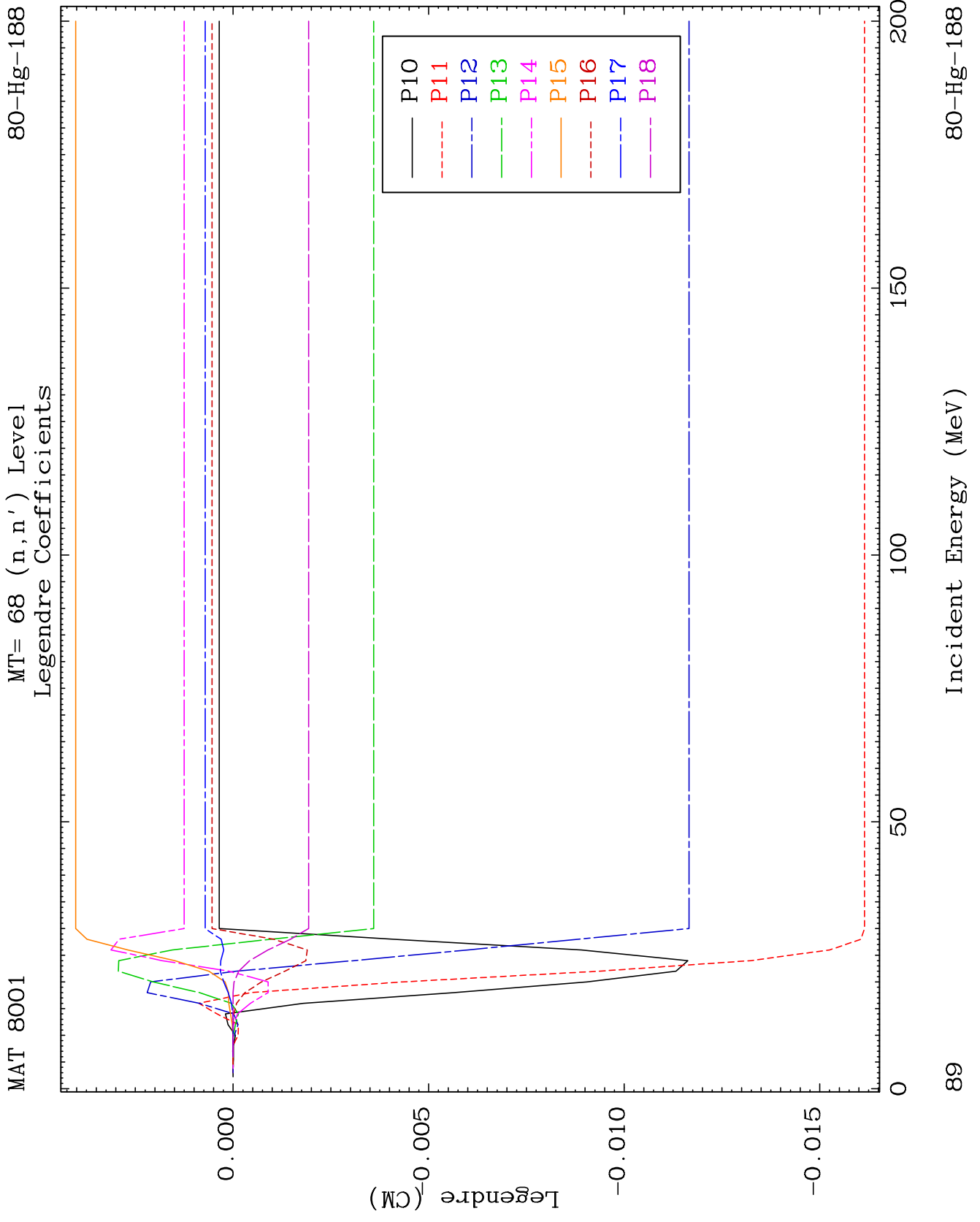
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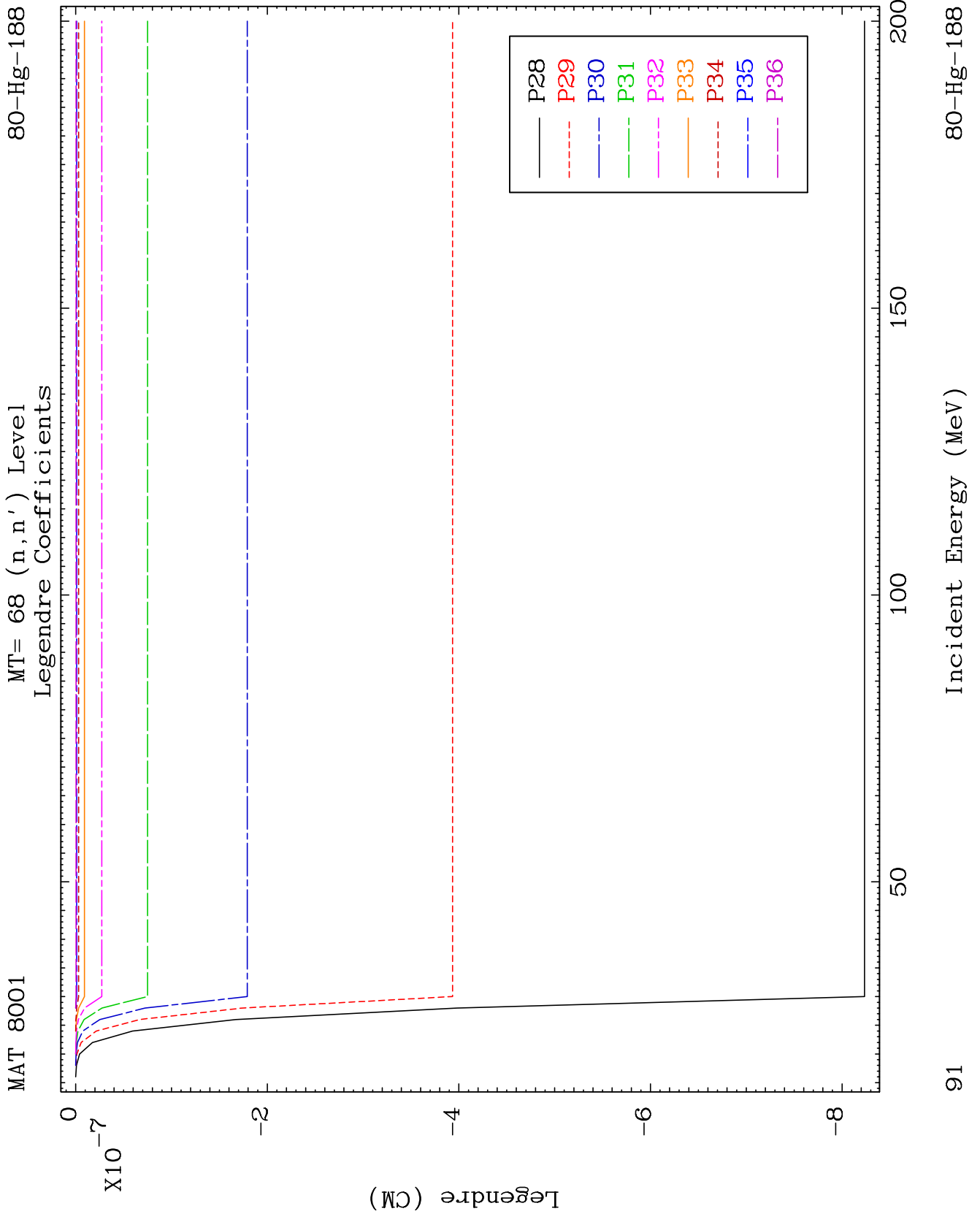


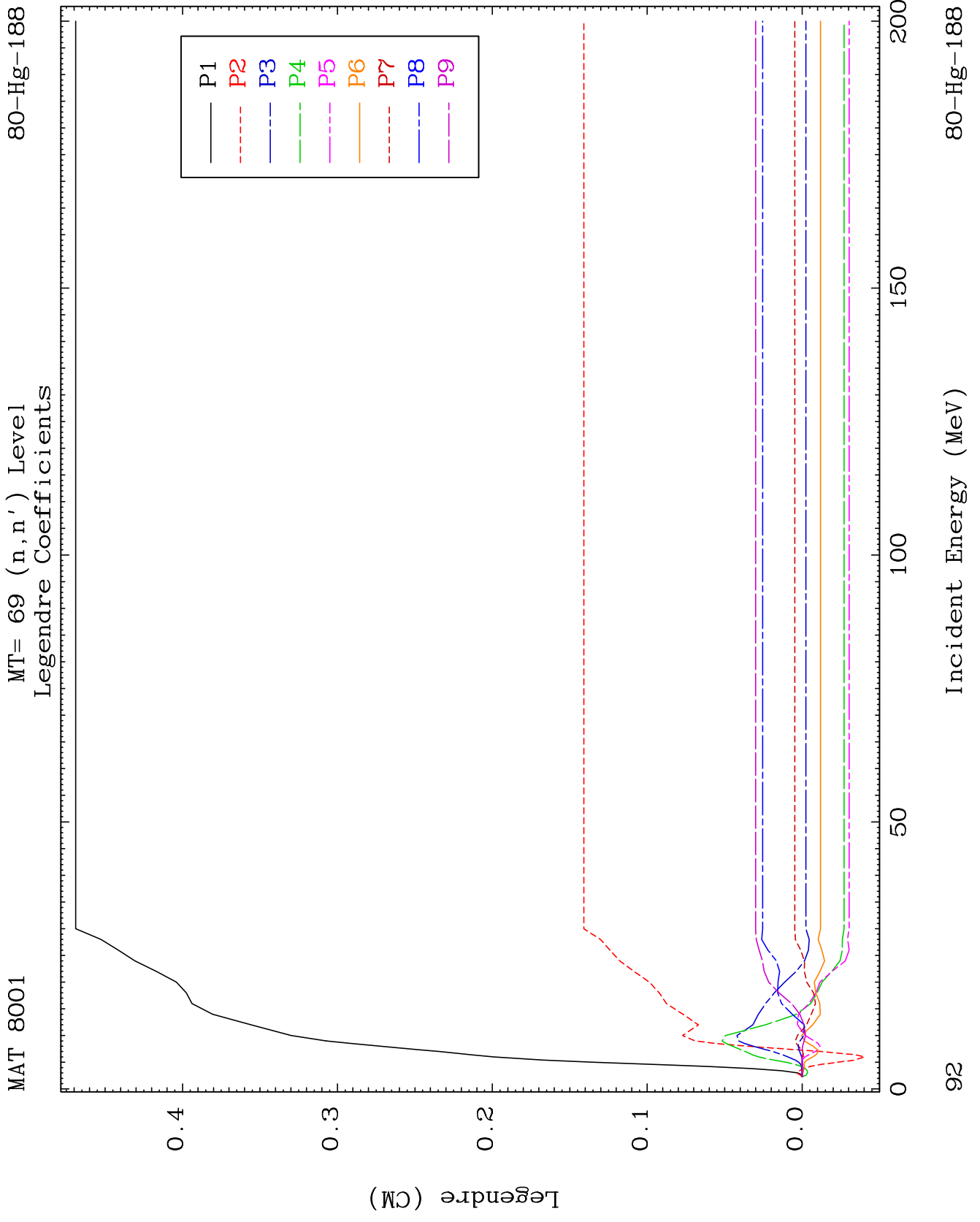
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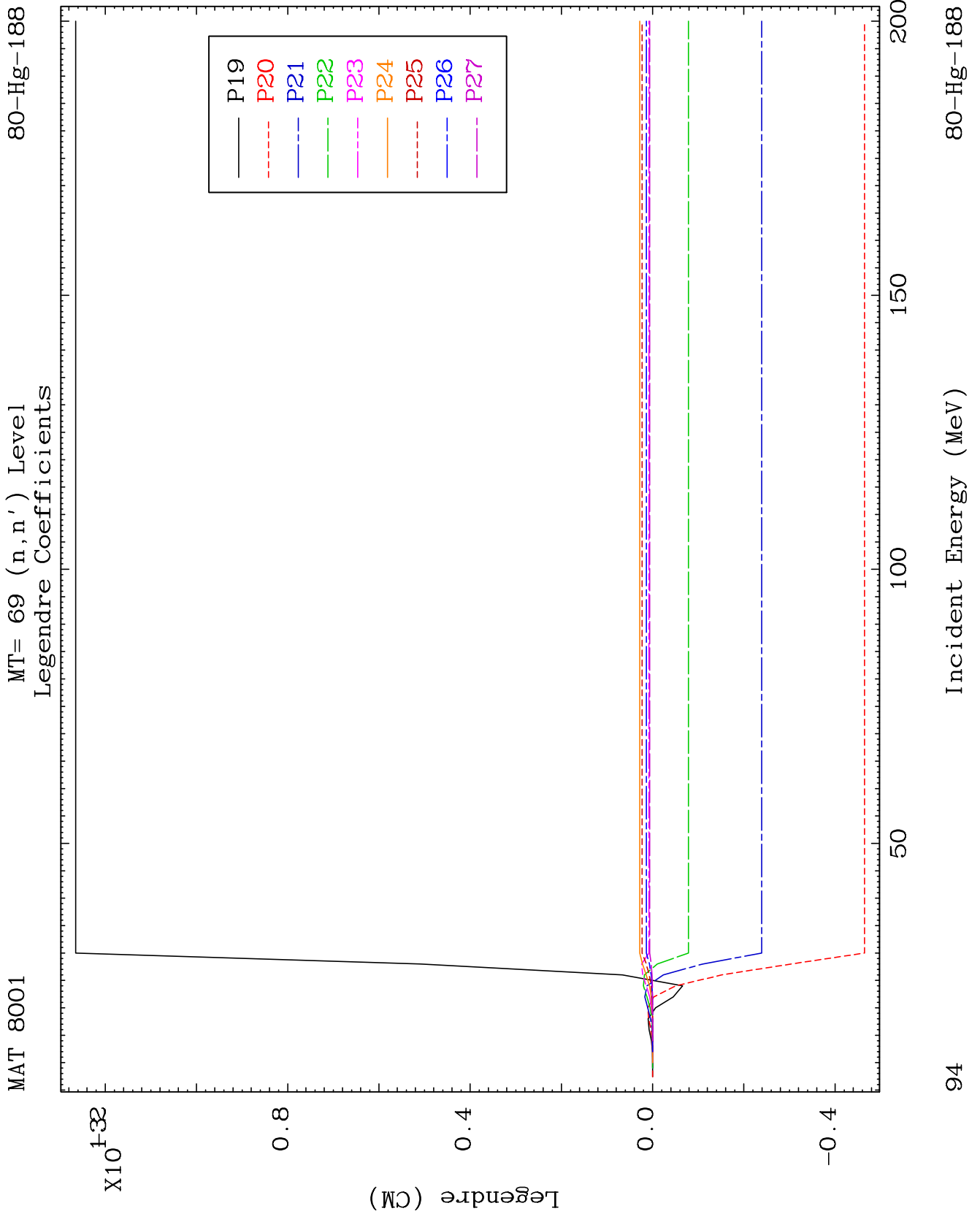
Incident Energy (MeV)

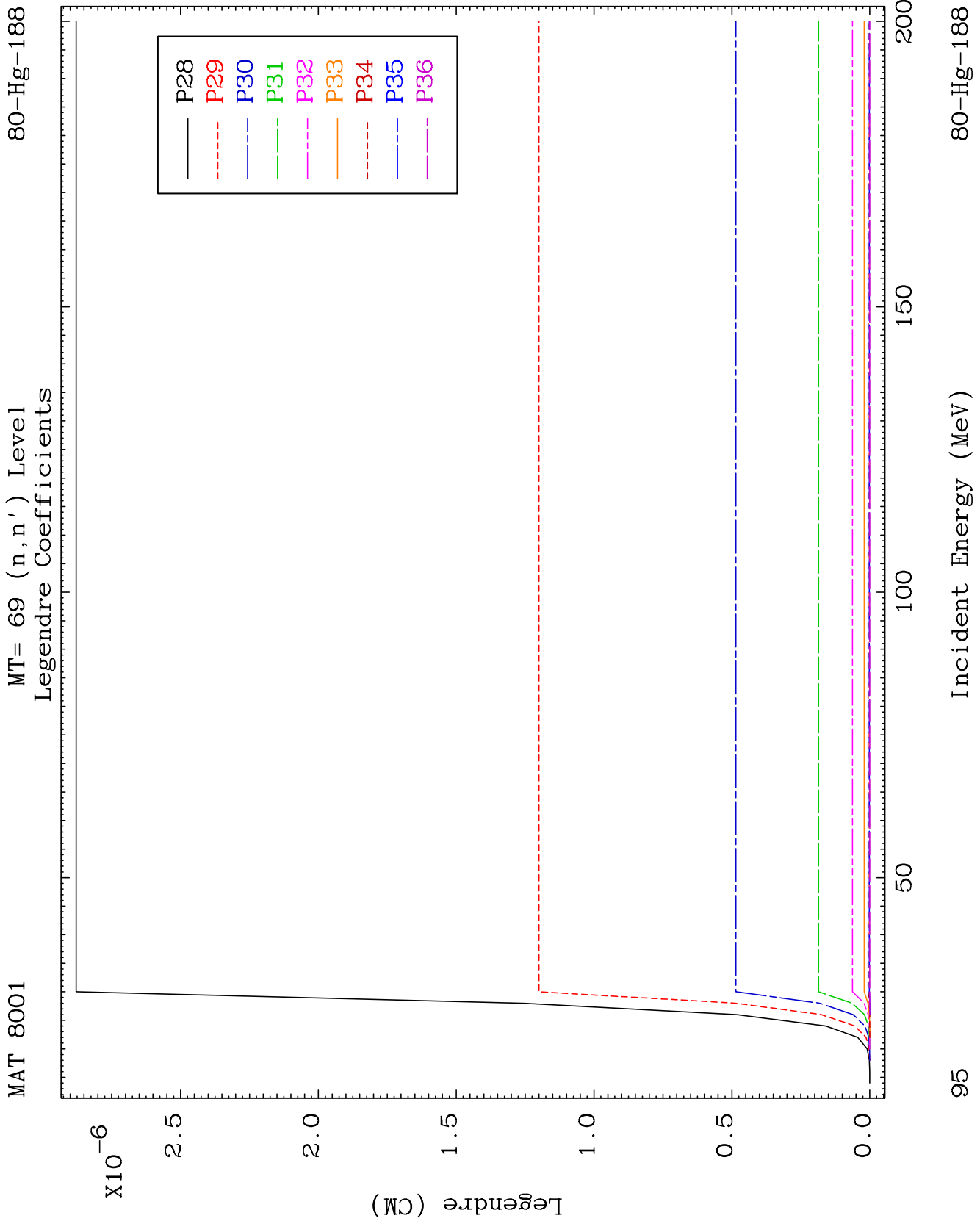
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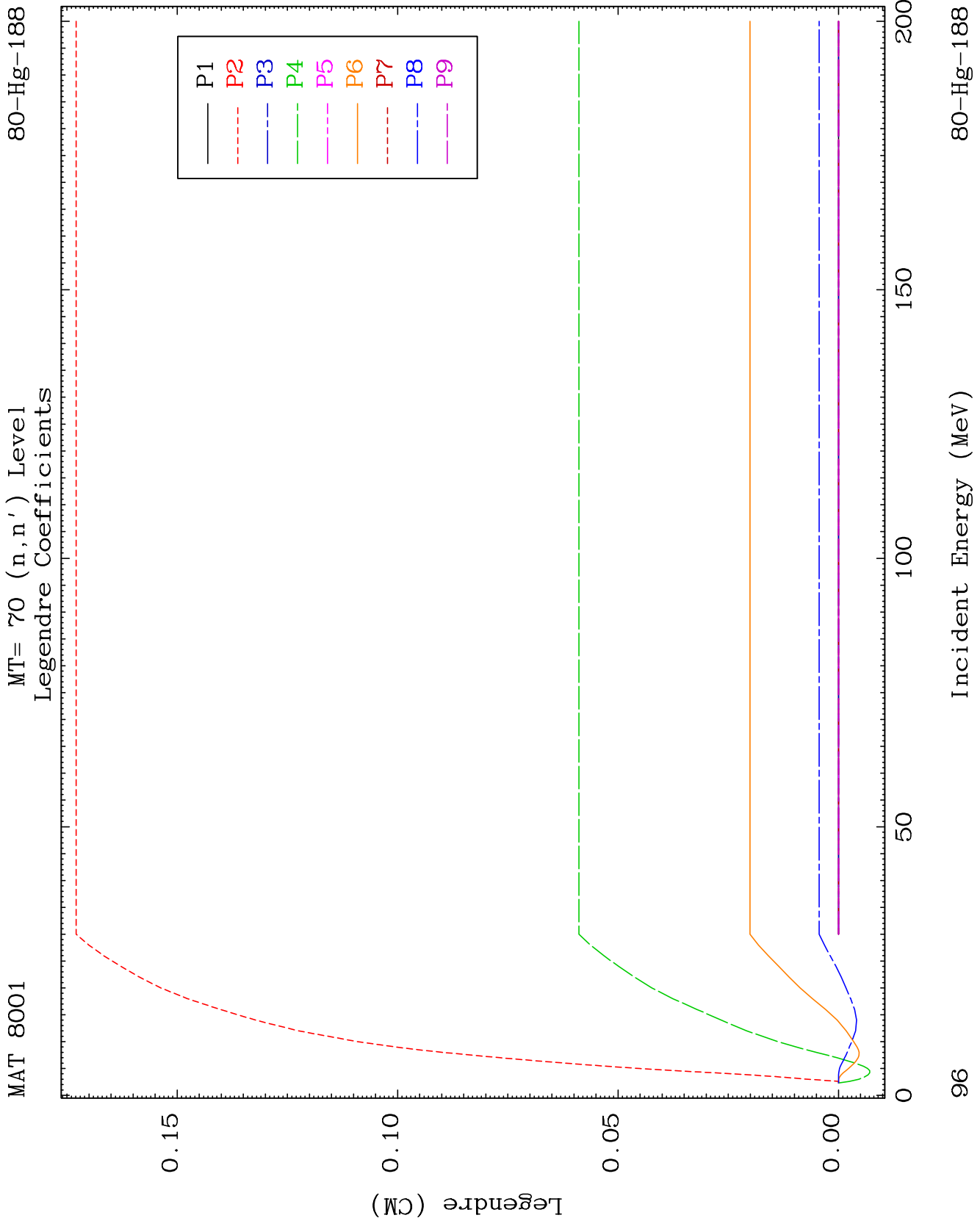








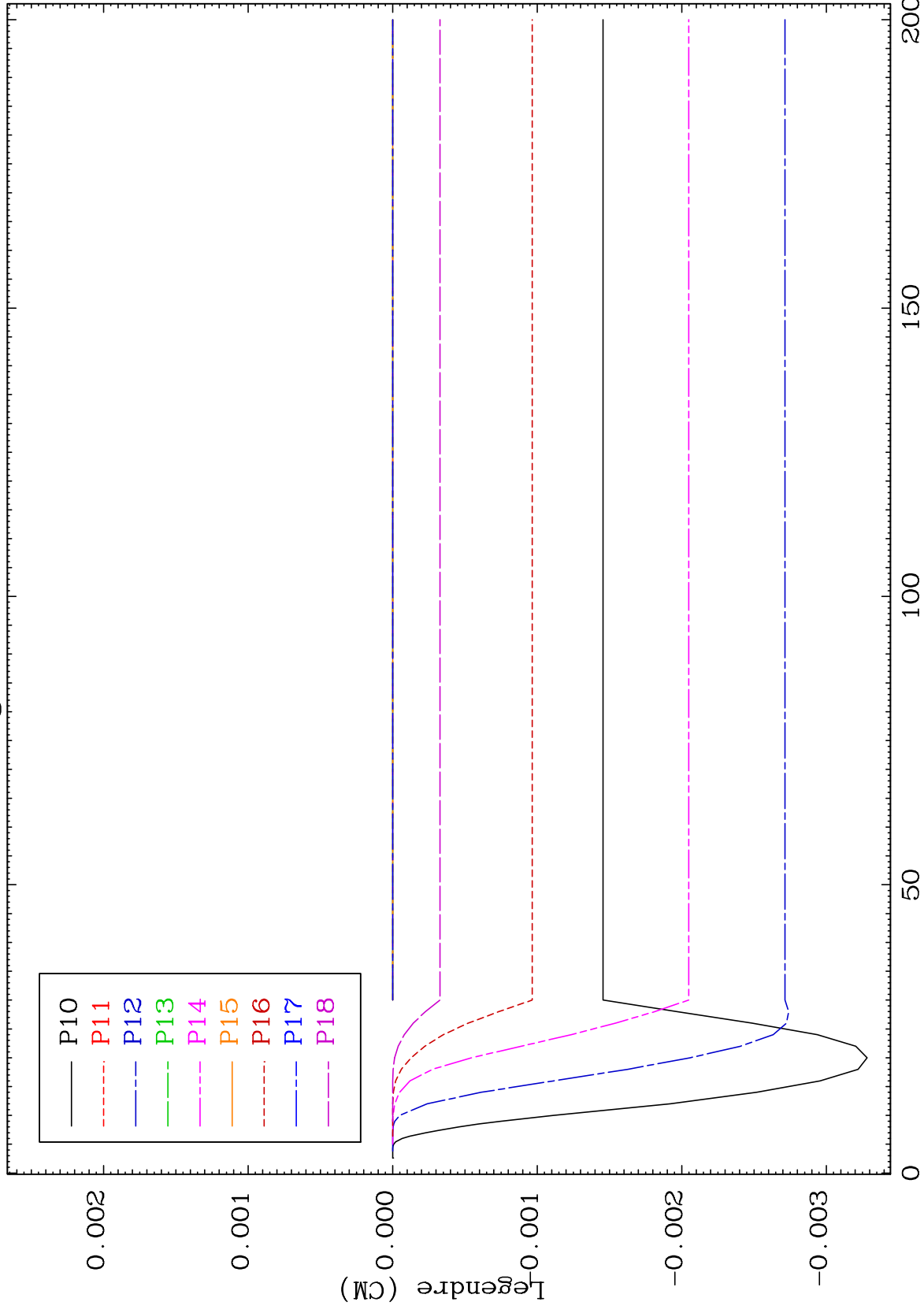




MAT 8001

MT= 70 (n,n') Level
Legendre Coefficients

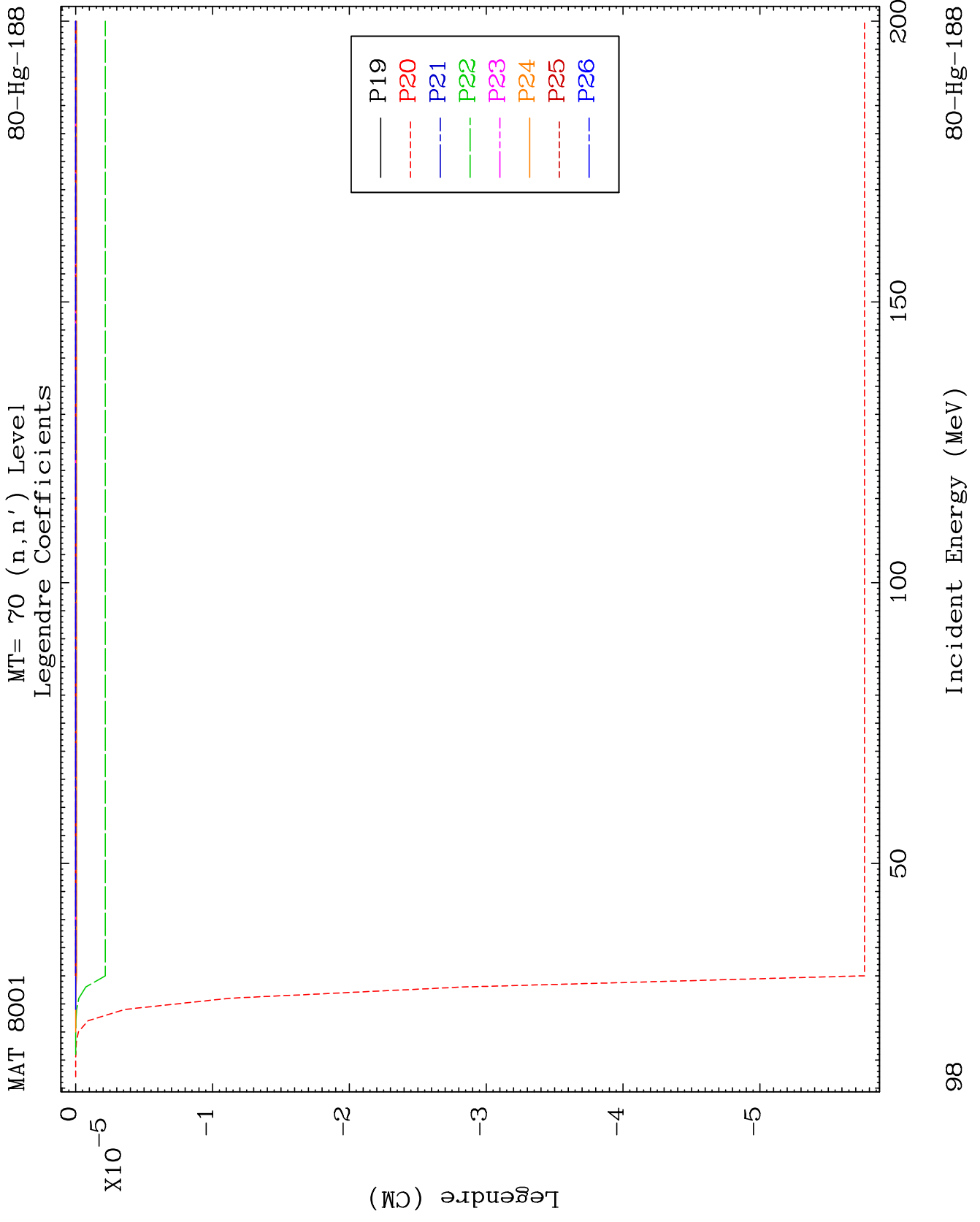
80-Hg-188

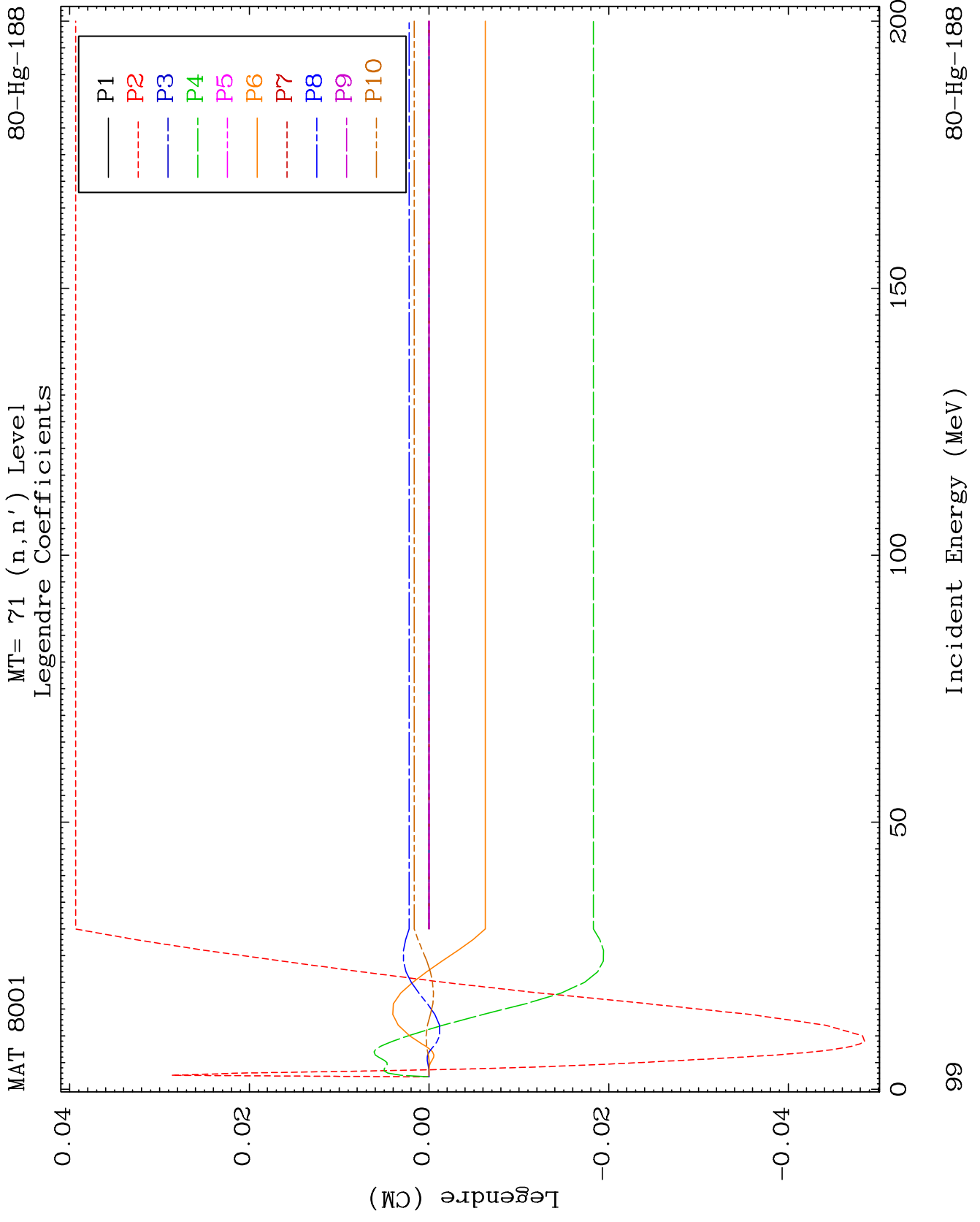


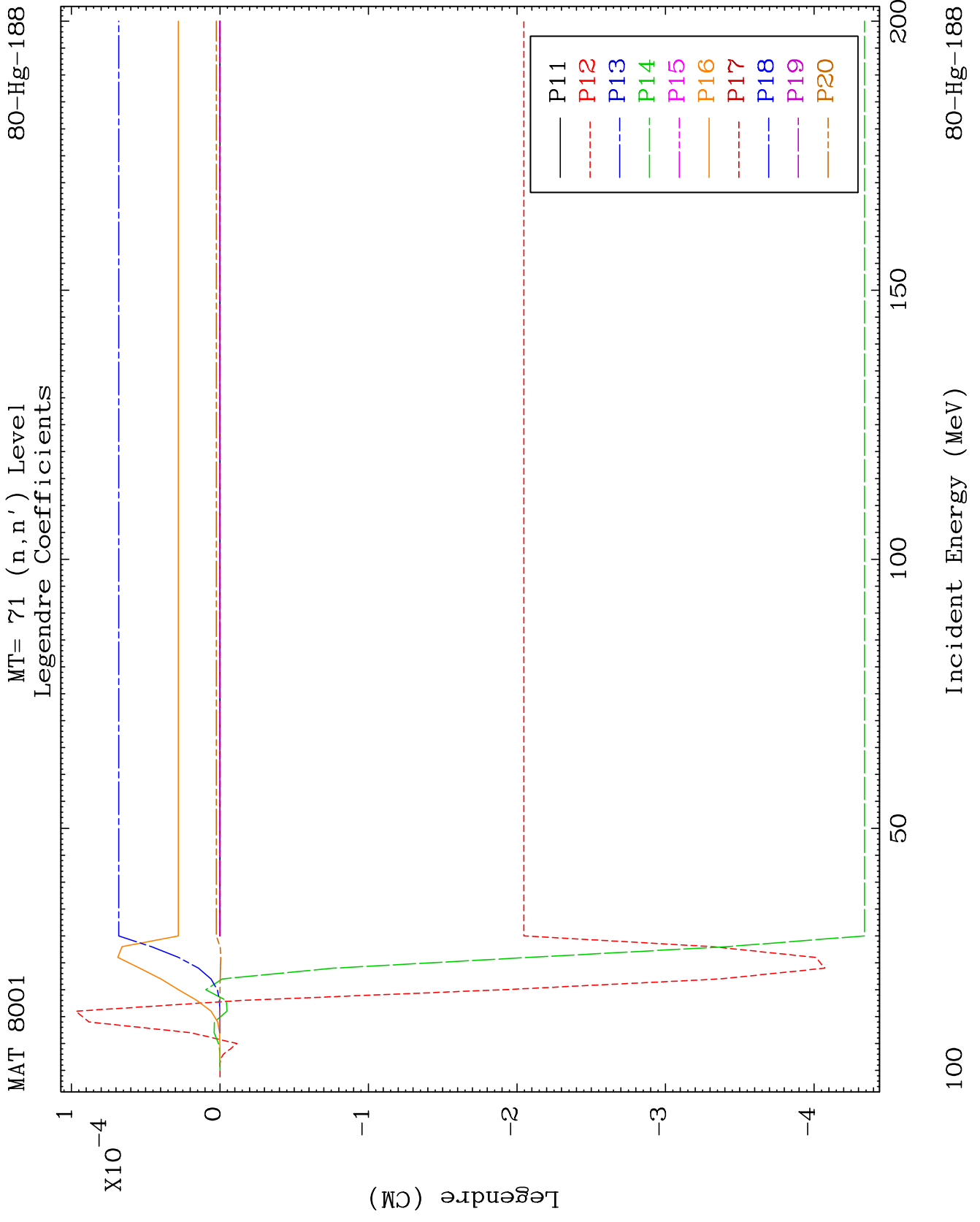
97

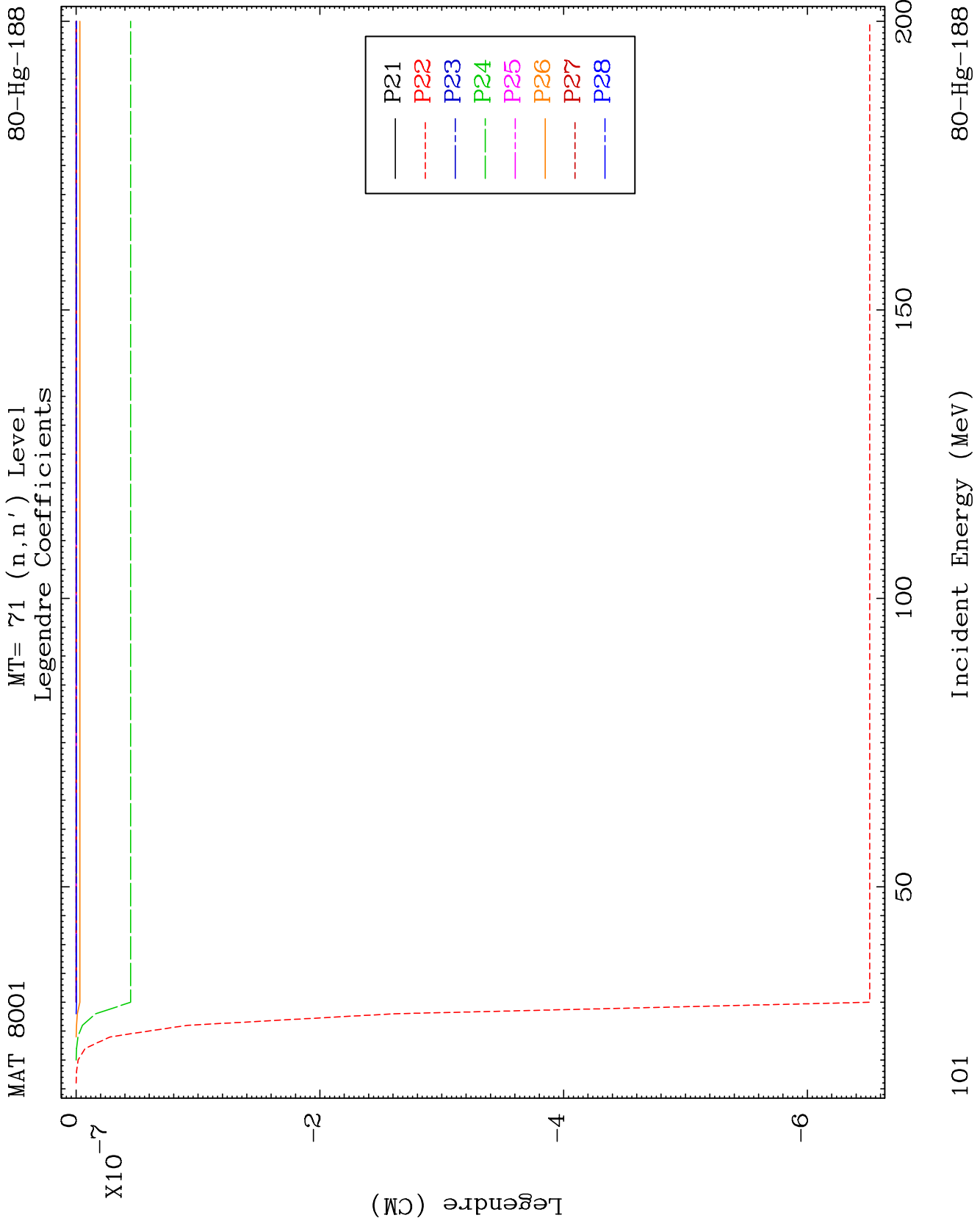
Incident Energy (MeV)

80-Hg-188





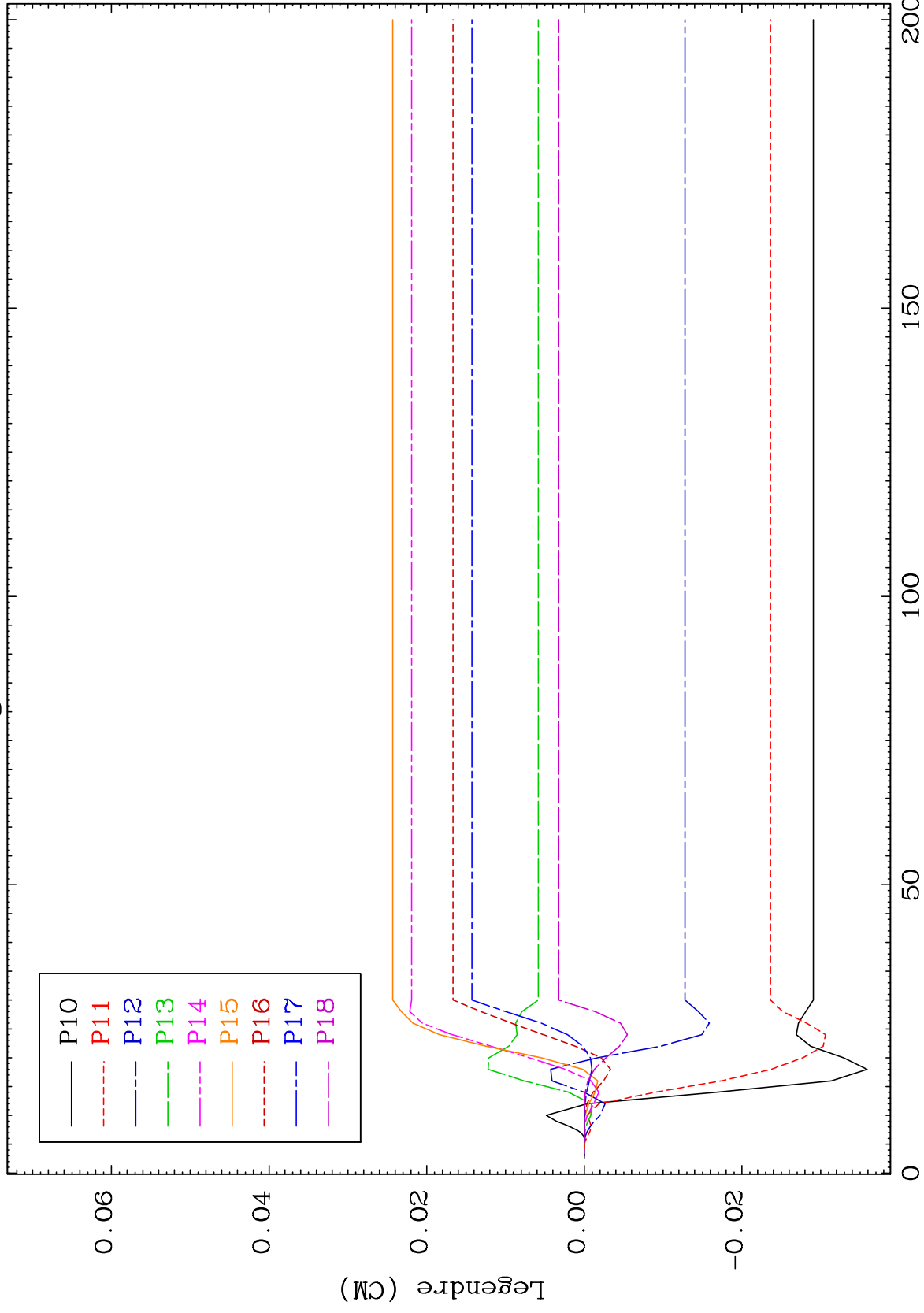




MAT 8001

MT= 72 (n,n') Level
Legendre Coefficients

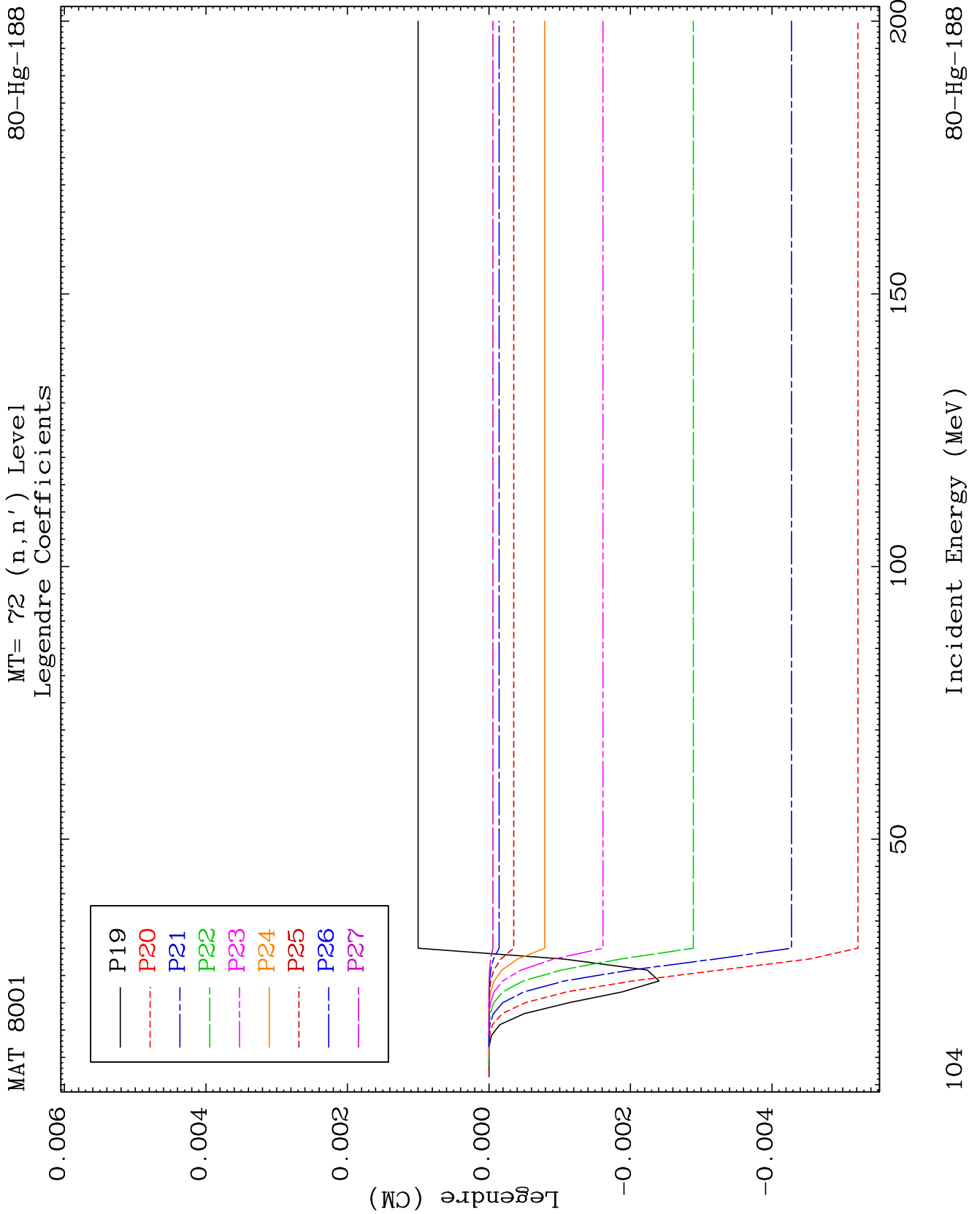
80-Hg-188

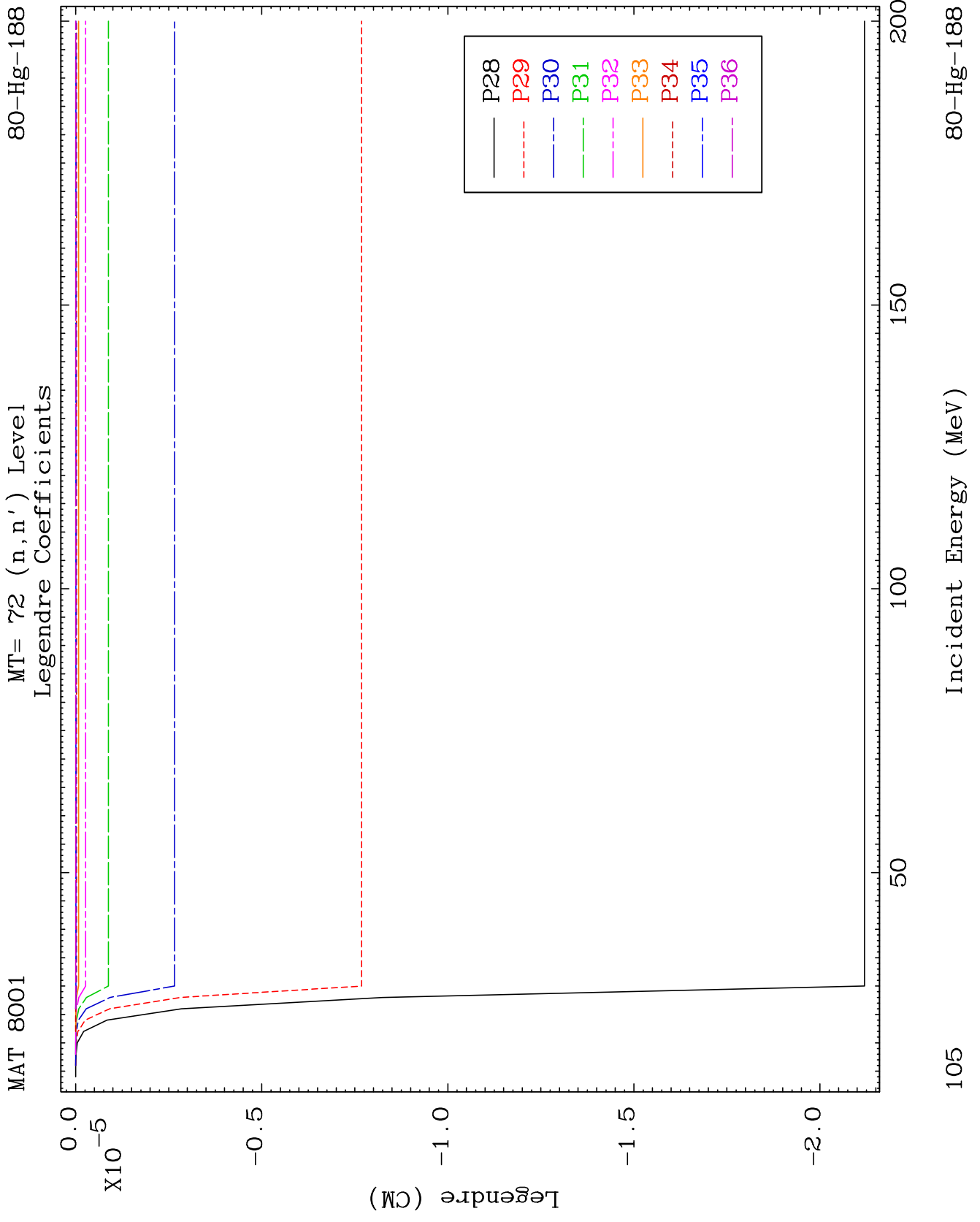


103

Incident Energy (MeV)

80-Hg-188



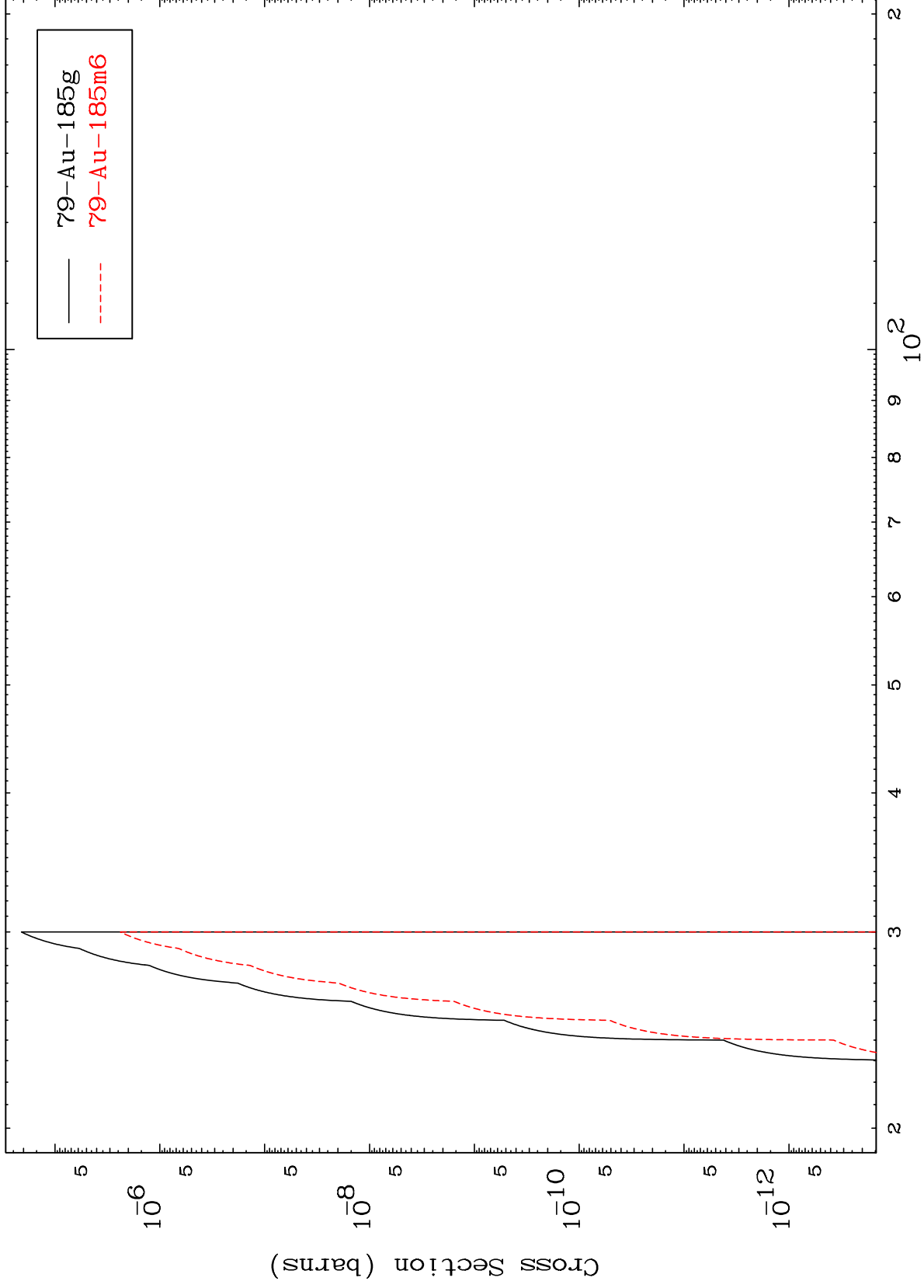


MAT 8001

(n,2n) d

80-Hg-188

Radionuclide Production Cross Section



106

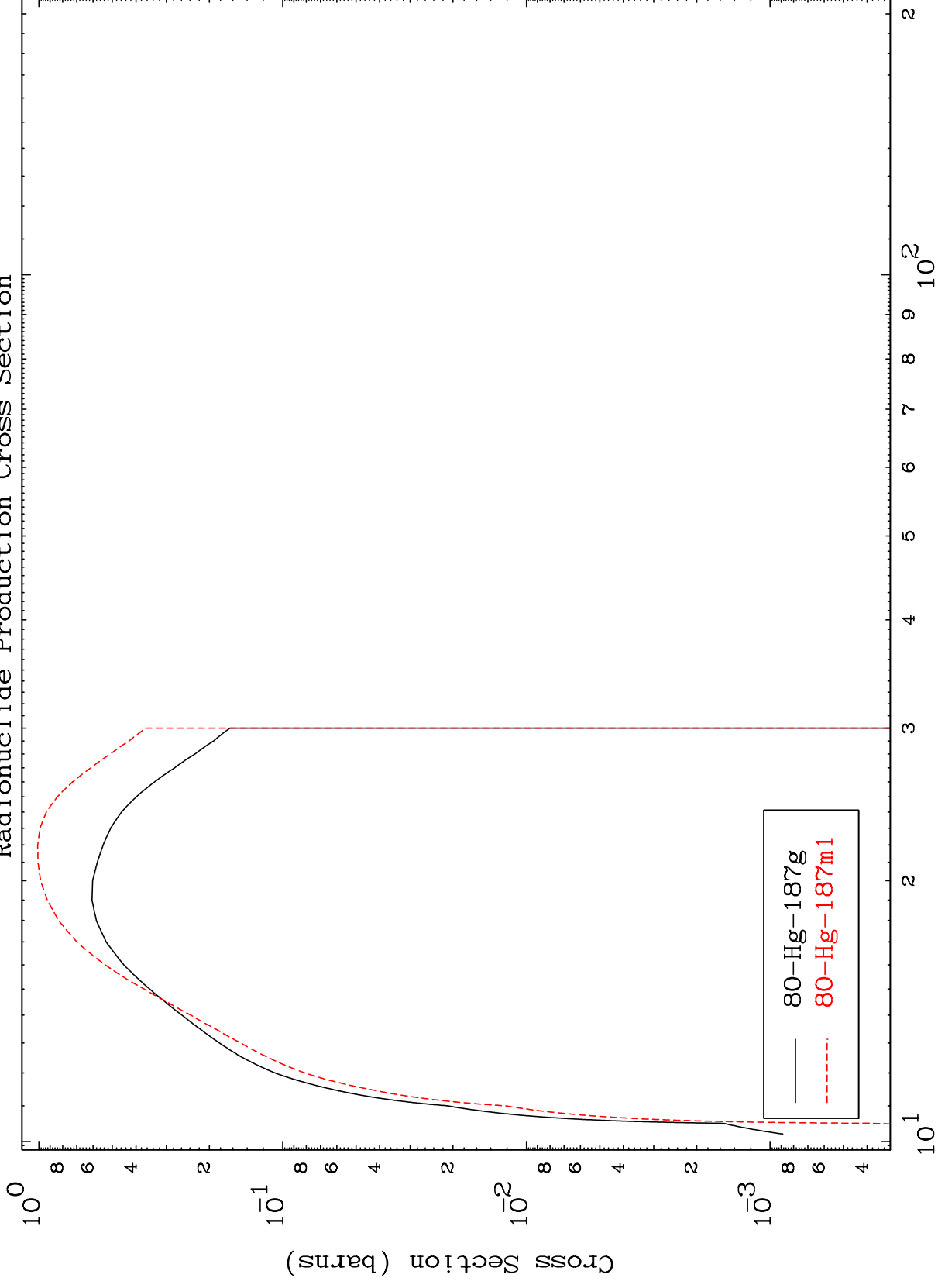
Incident Energy (MeV)

80-Hg-188

MAT 8001

80-Hg-188

(n,2n)
Radionuclide Production Cross Section



80-Hg-188

Incident Energy (MeV)

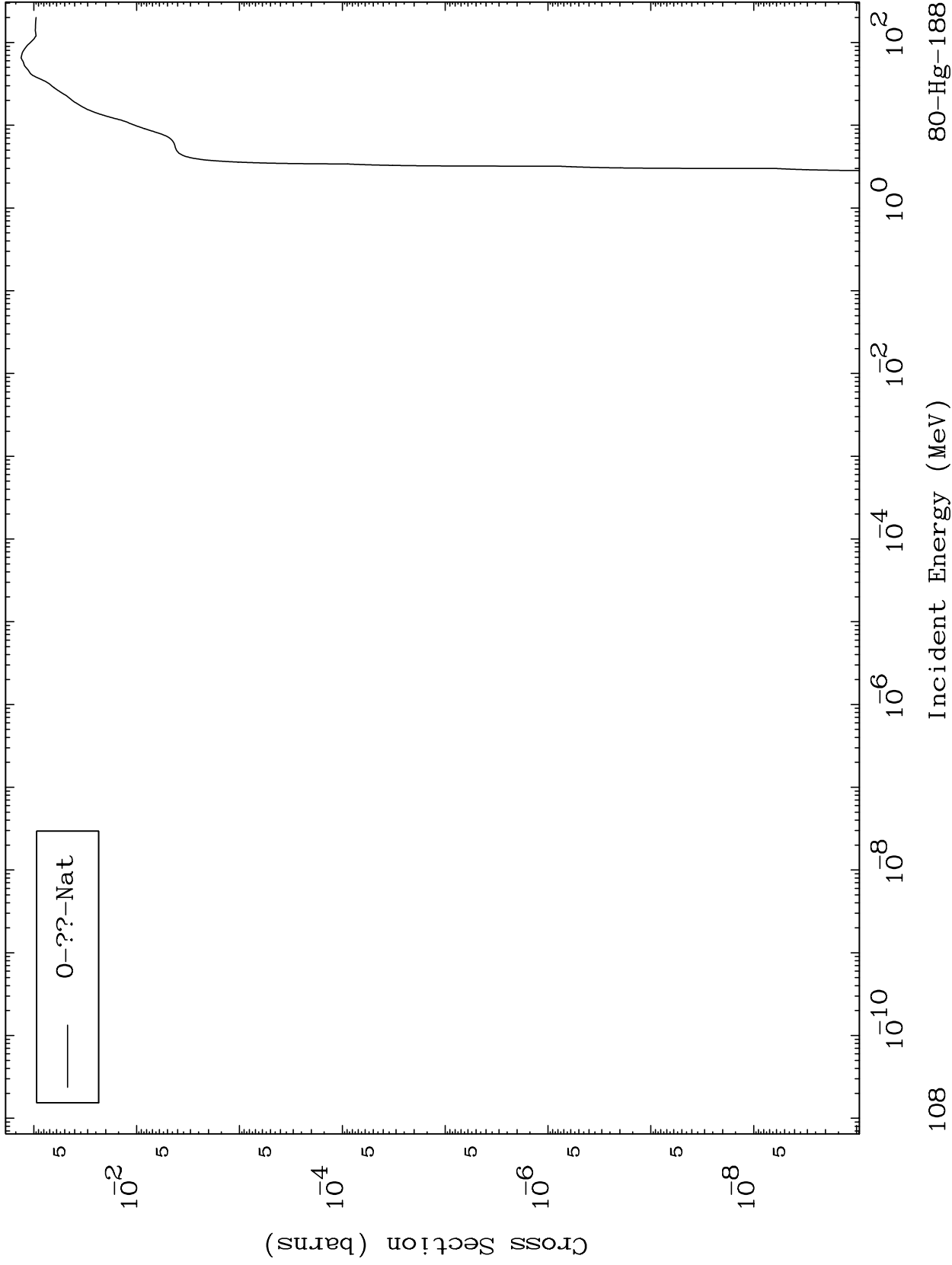
107

MAT 8001

Fission

80-Hg-188

Radionuclide Production Cross Section



108

Incident Energy (MeV)

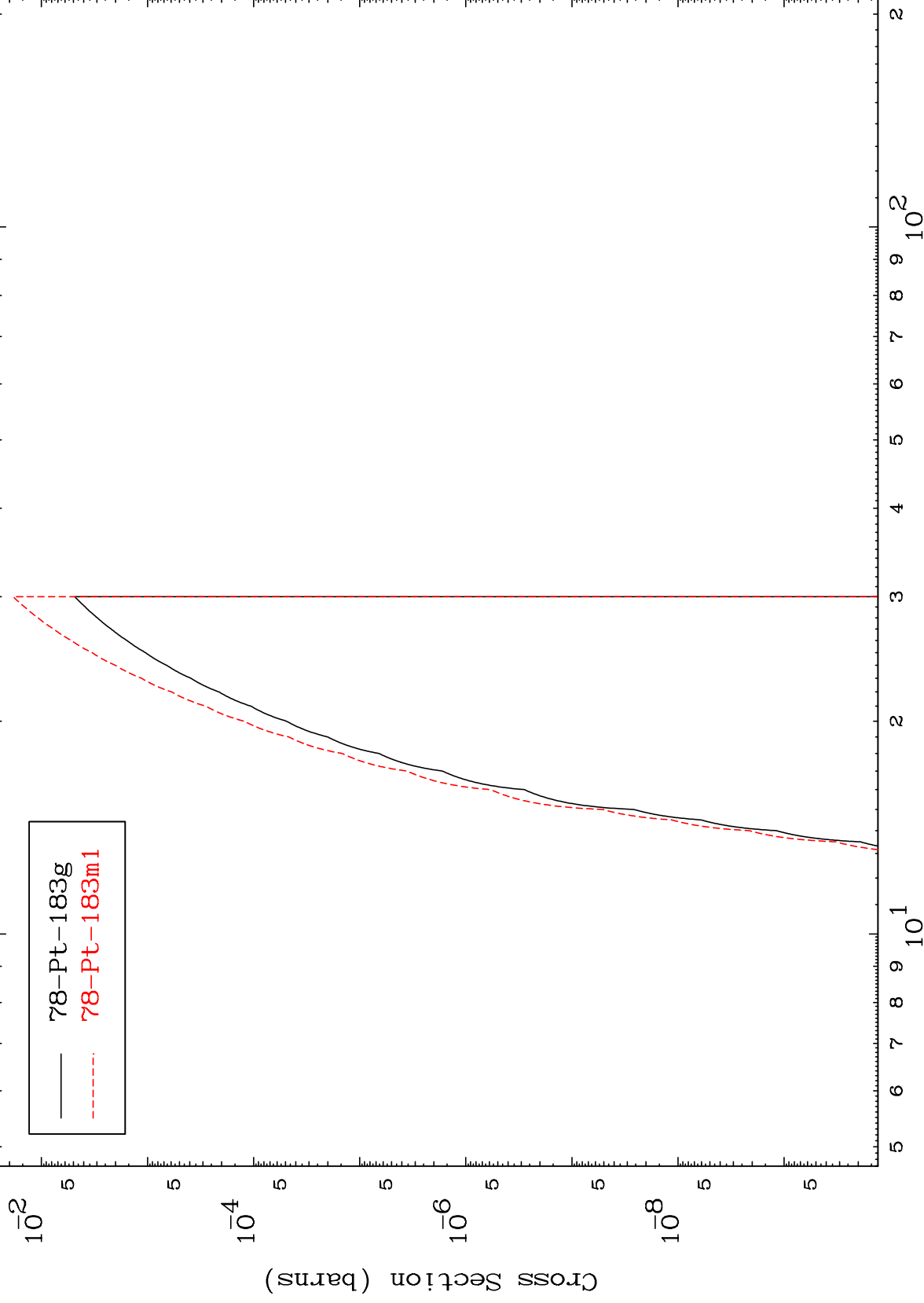
80-Hg-188

MAT 8001

$(n, 2n) \alpha$

80-Hg-188

Radionuclide Production Cross Section



109

Incident Energy (MeV)

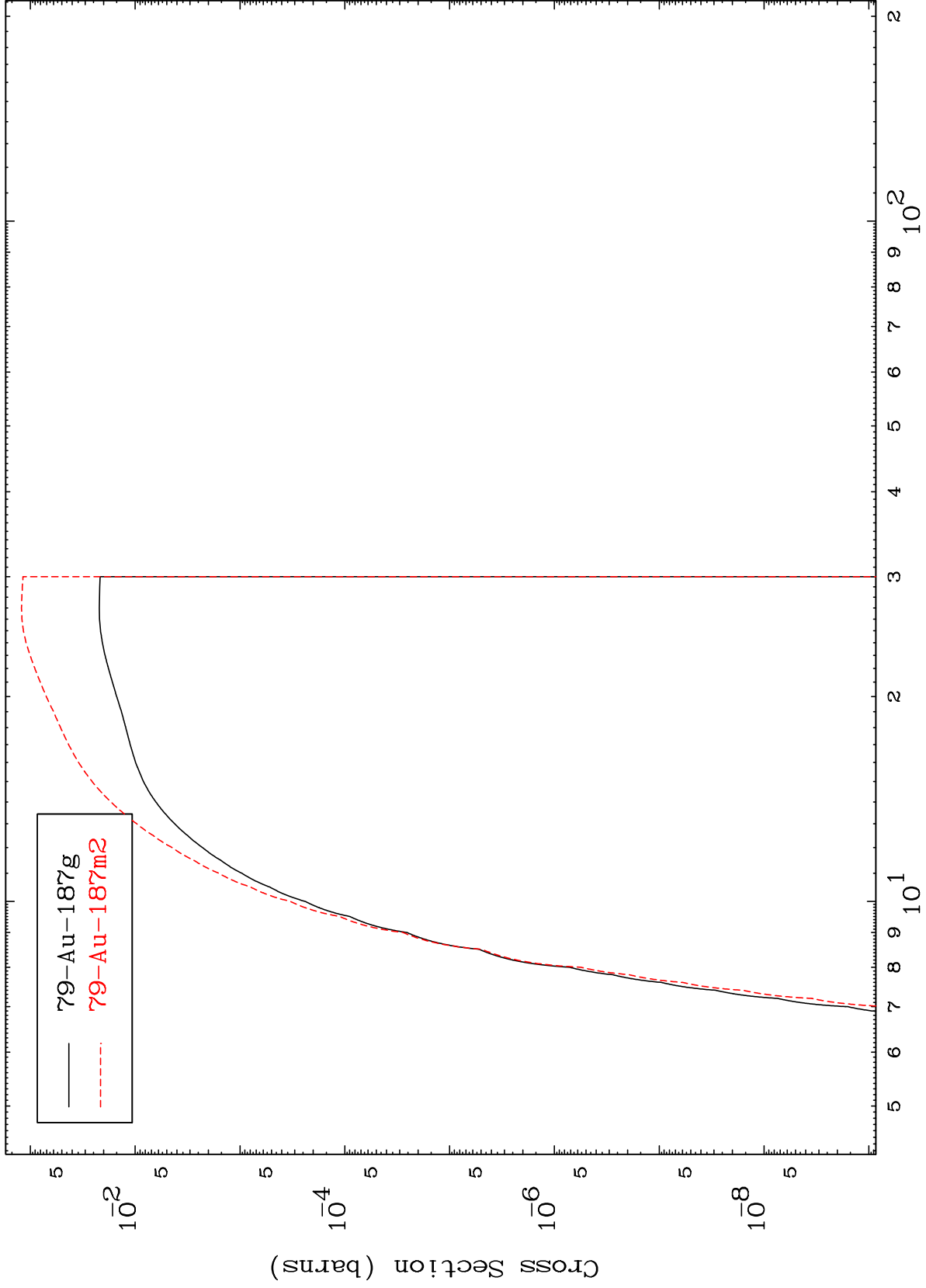
80-Hg-188

MAT 8001

(n,n') p

80-Hg-188

Radionuclide Production Cross Section



110

Incident Energy (MeV)

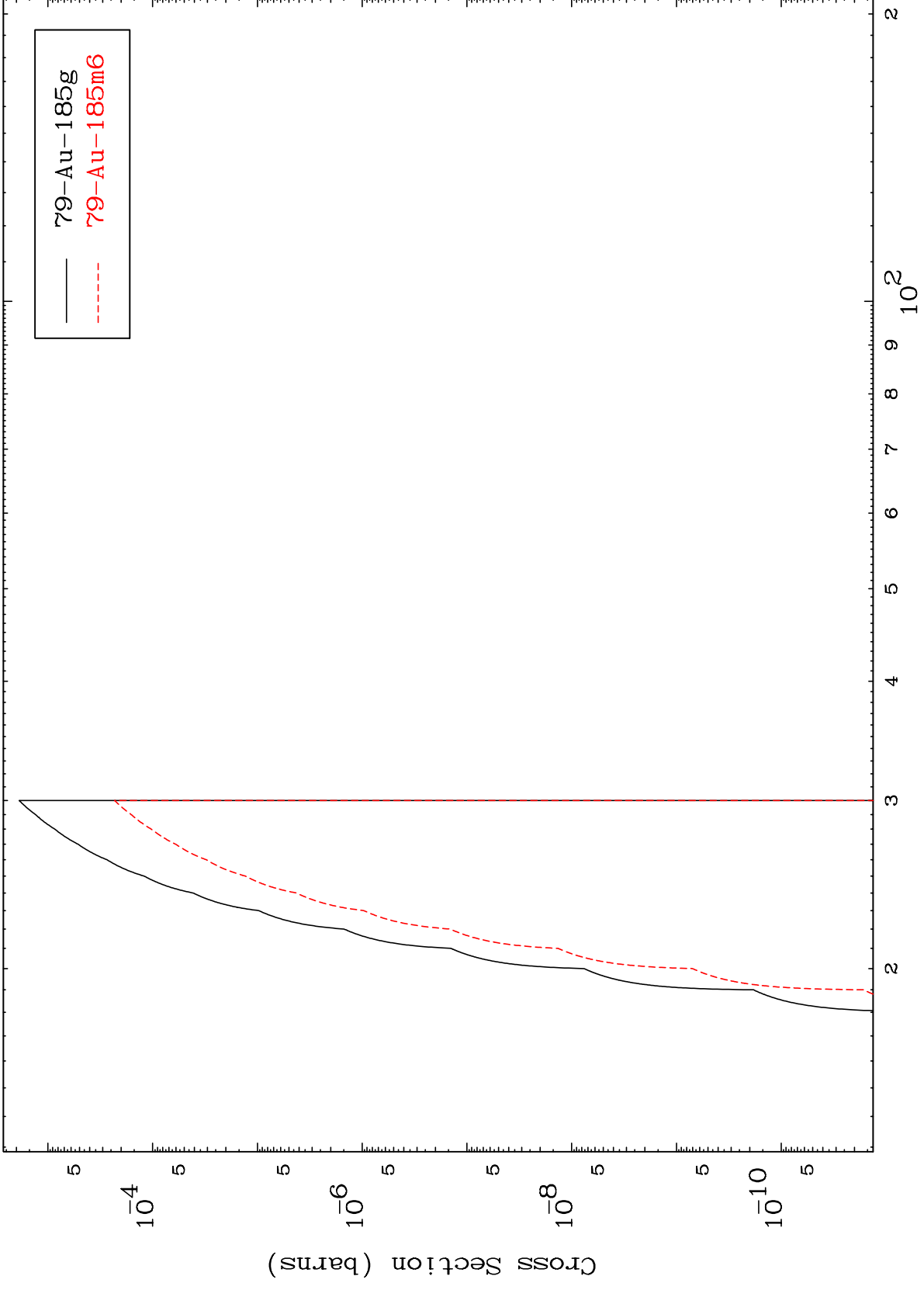
80-Hg-188

MAT 8001

(n,n') t

80-Hg-188

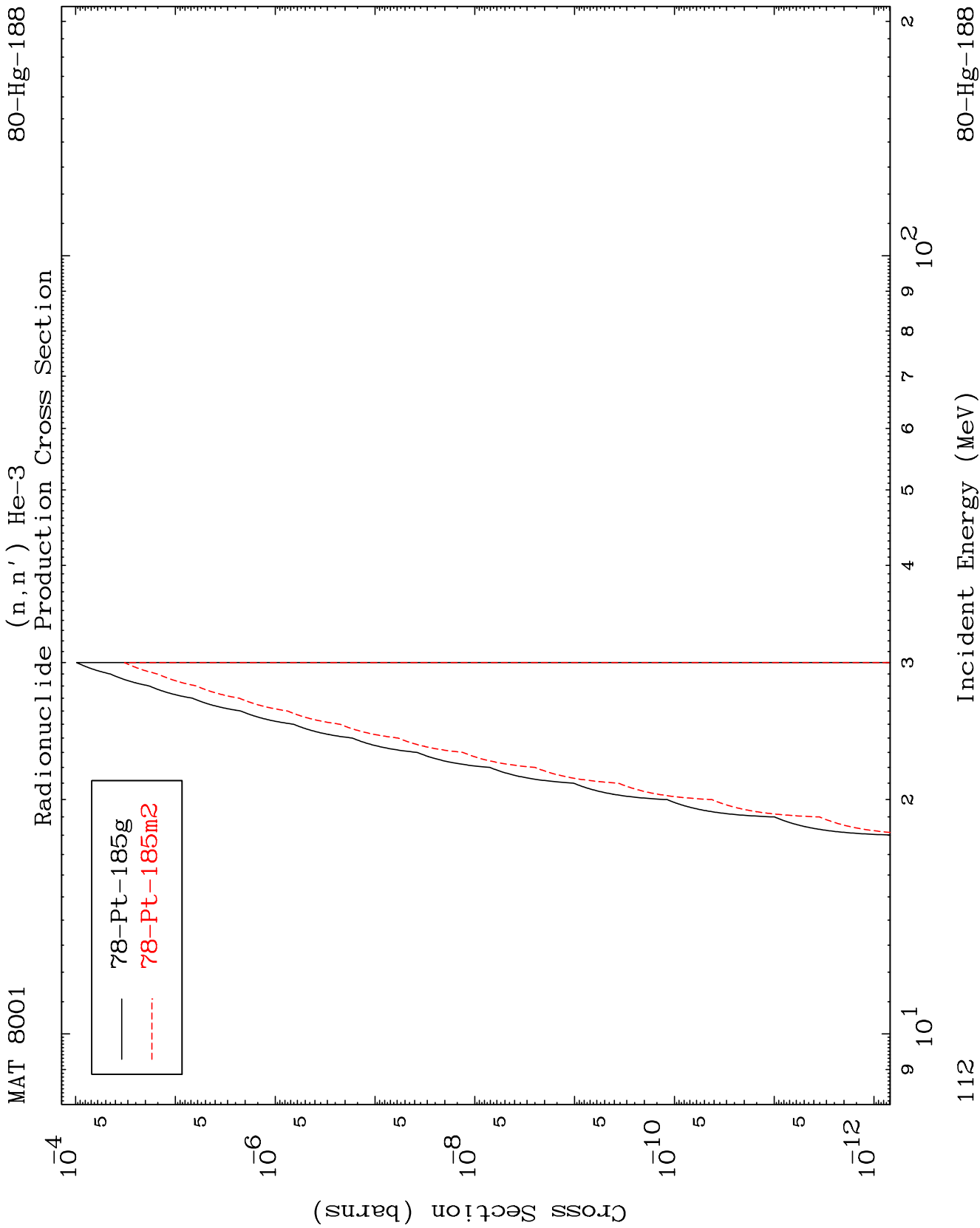
Radionuclide Production Cross Section



111

Incident Energy (MeV)

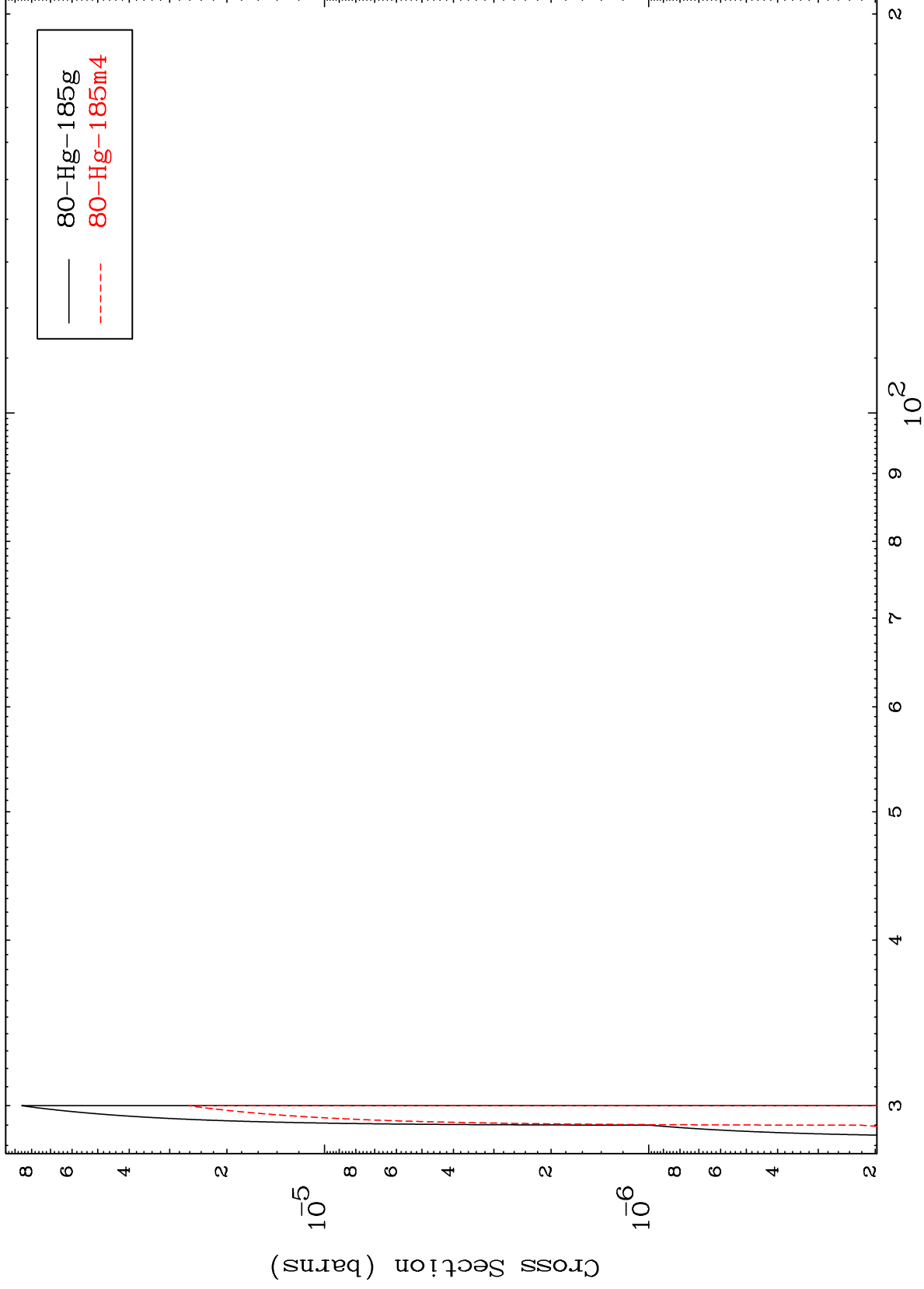
80-Hg-188



MAT 8001

80-Hg-188

(n,4n)
Radionuclide Production Cross Section



80-Hg-188

Incident Energy (MeV)

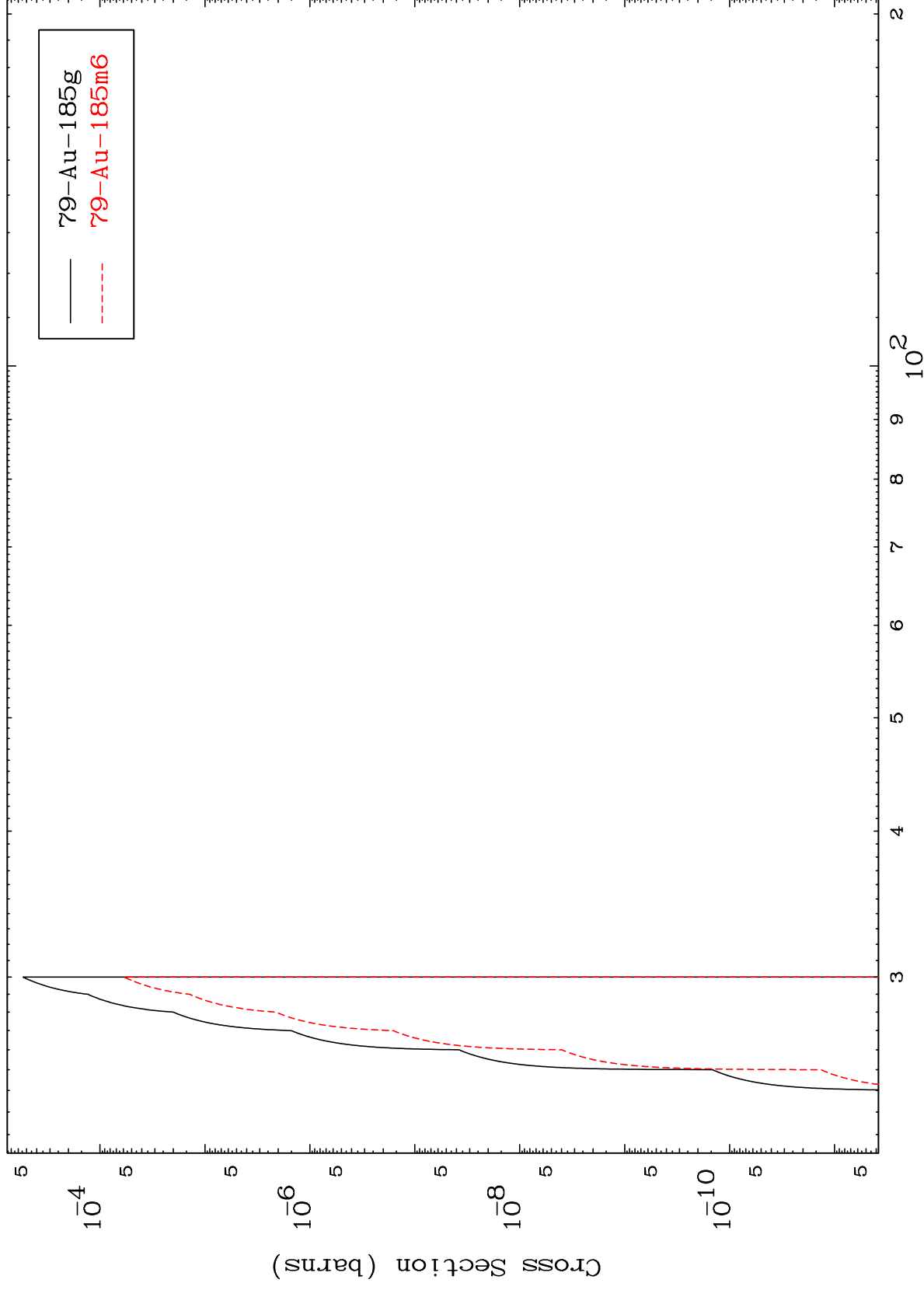
113

MAT 80001

(n,3n) p

80-Hg-188

Radionuclide Production Cross Section



114

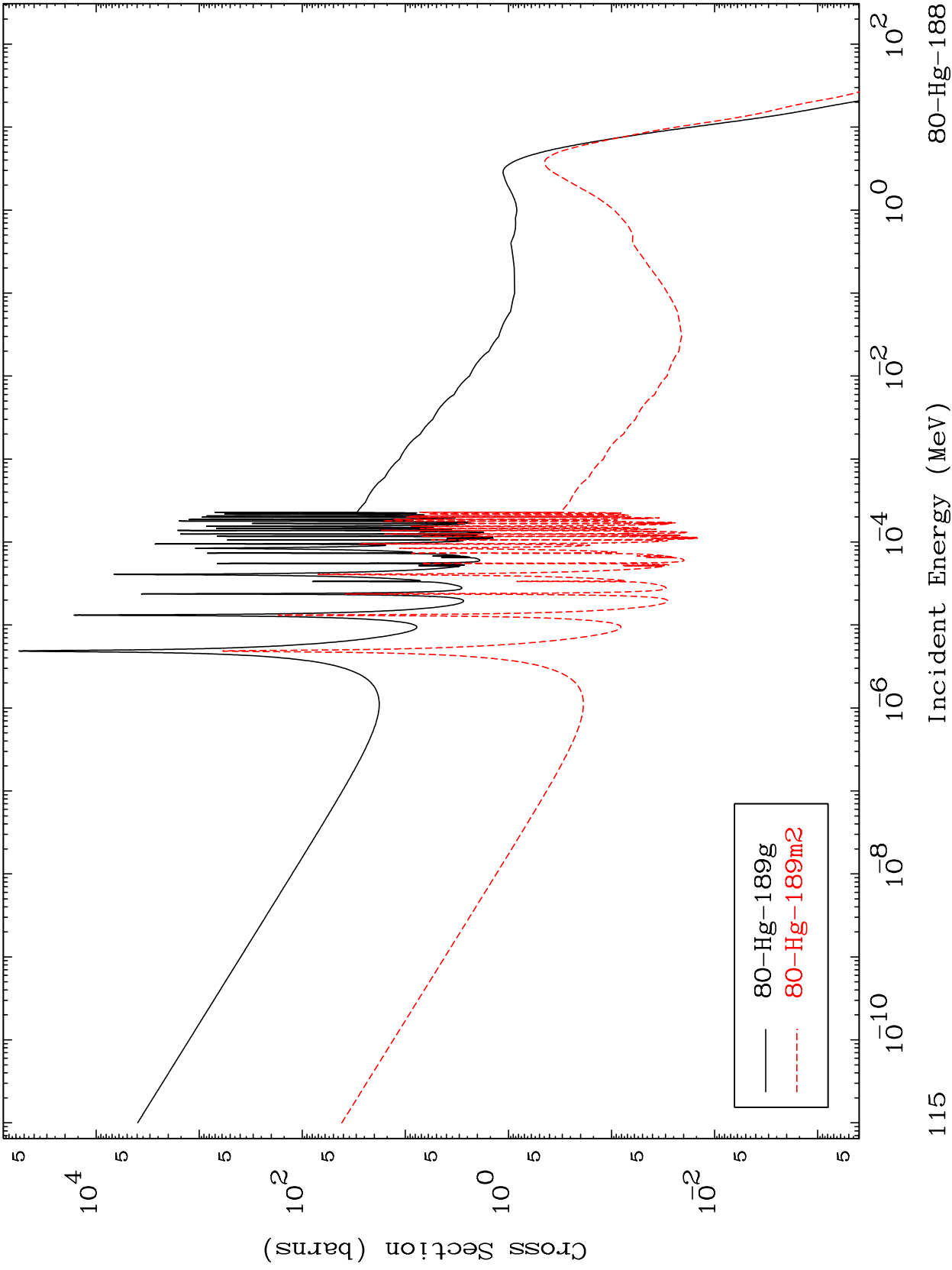
Incident Energy (MeV)

80-Hg-188

MAT 8001

80-Hg-188

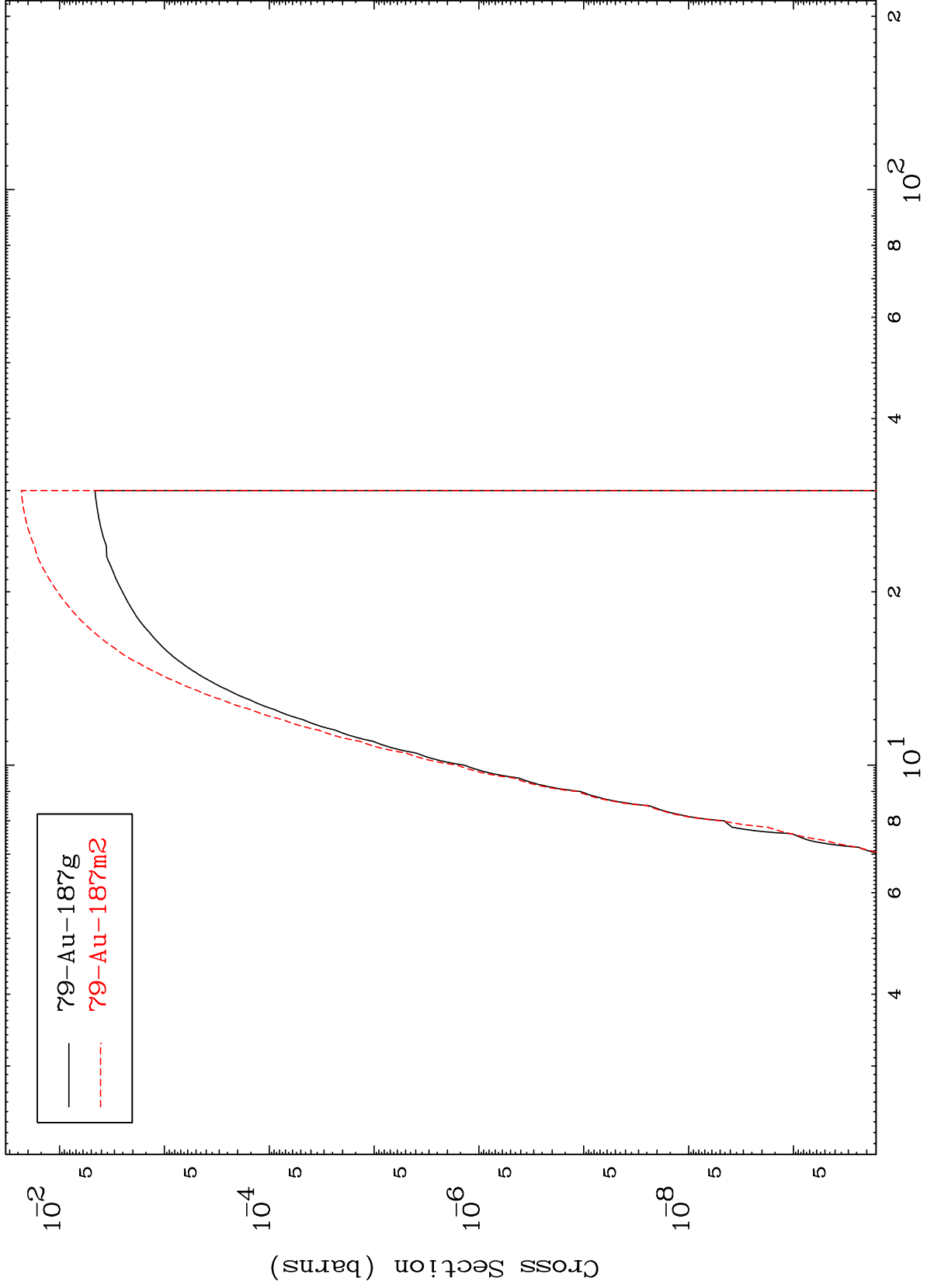
(n, γ)
Radionuclide Production Cross Section



MAT 8001

80-Hg-188

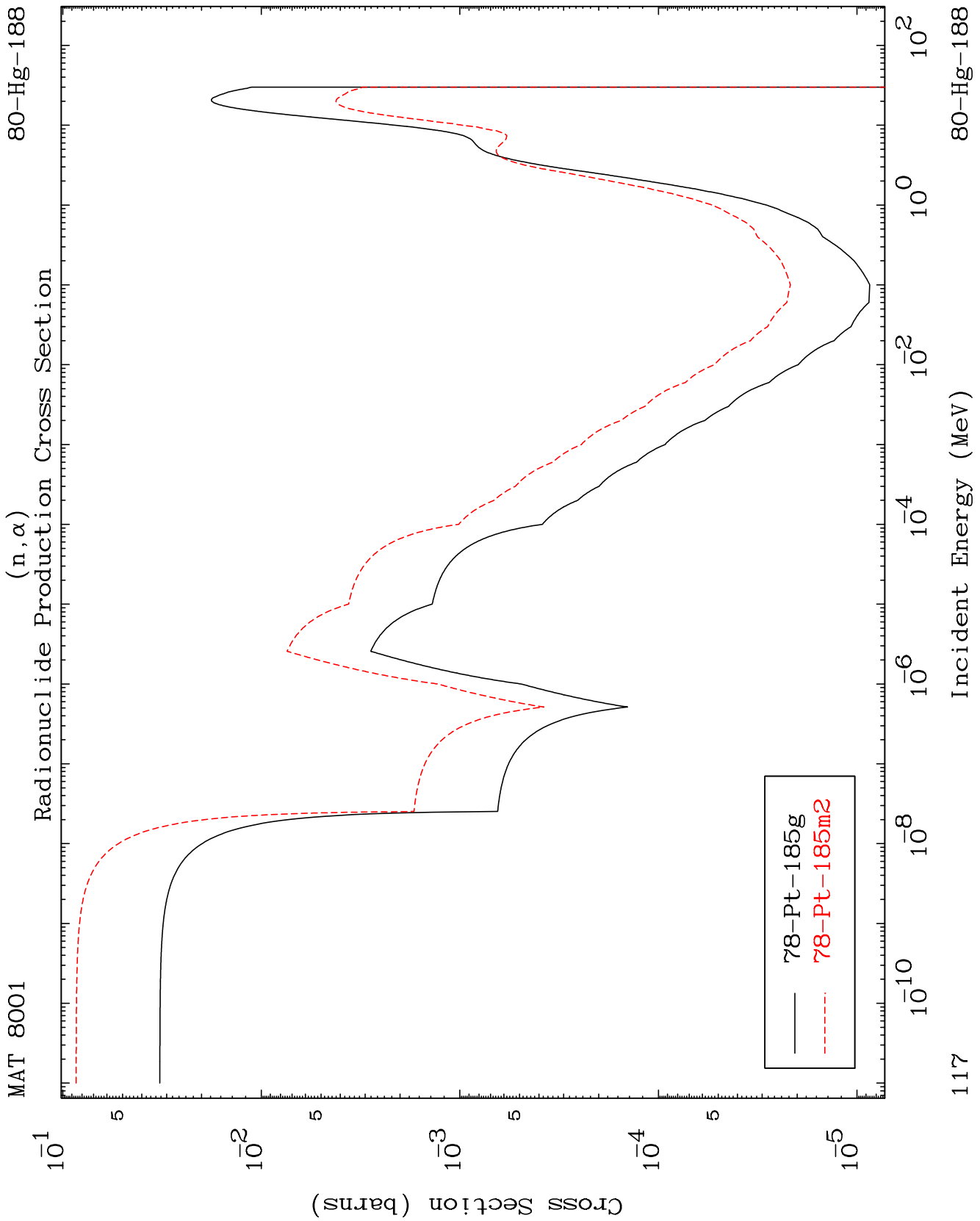
(n,d)
Radionuclide Production Cross Section



116

80-Hg-188

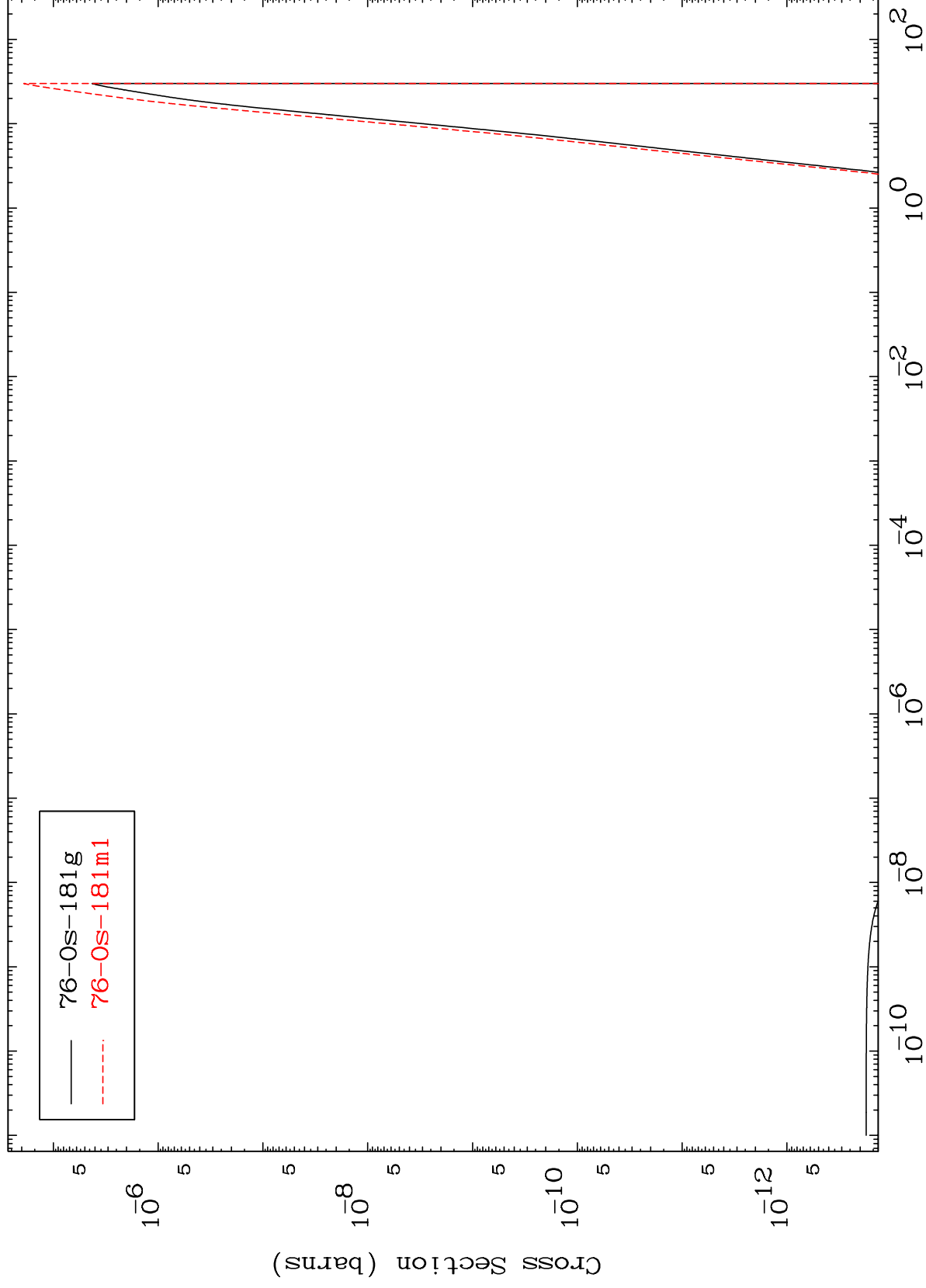
Incident Energy (MeV)



MAT 8001

80-Hg-188

(n,2α)
Radionuclide Production Cross Section



118

Incident Energy (MeV)

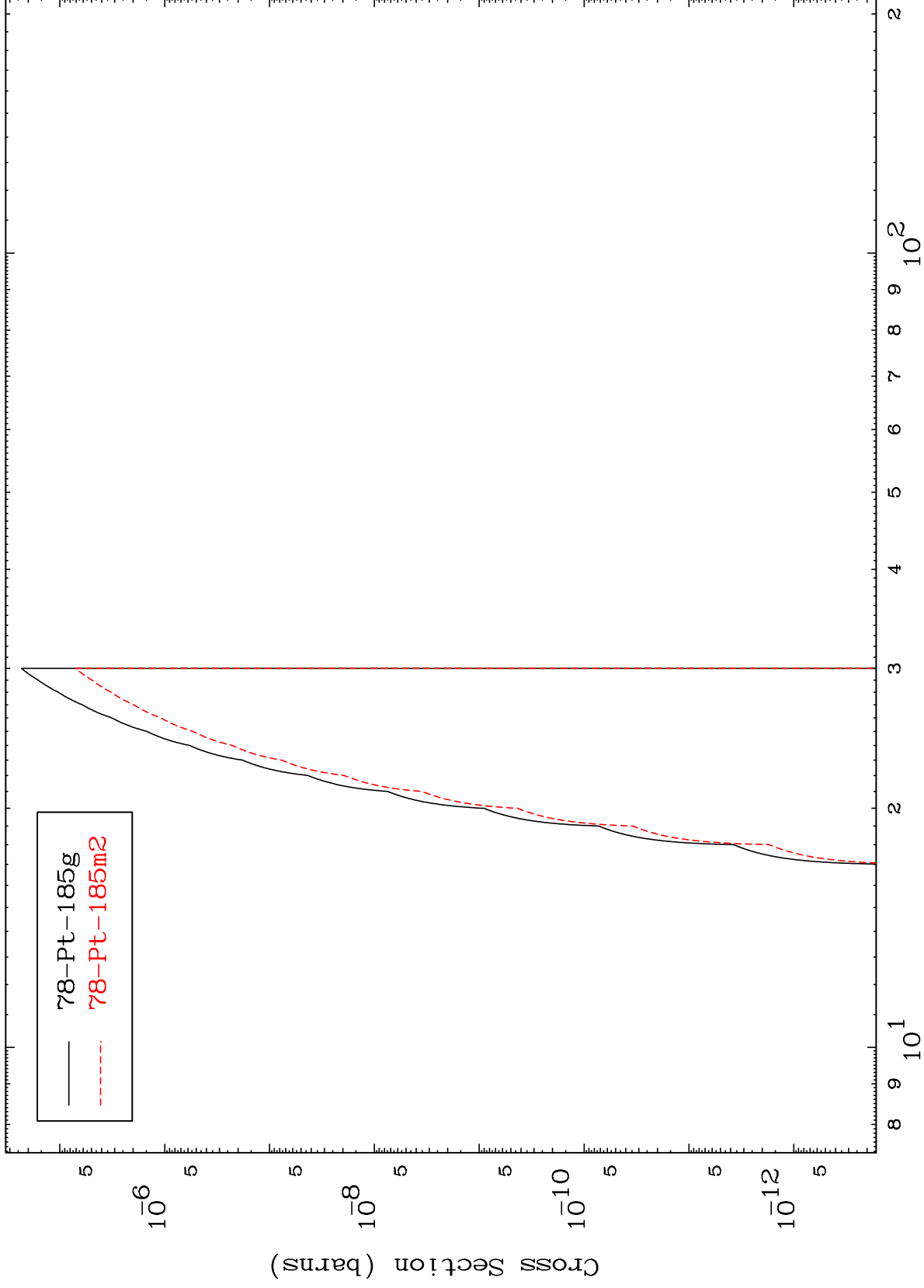
80-Hg-188

MAT 8001

(n,p) t

80-Hg-188

Radionuclide Production Cross Section



119

Incident Energy (MeV)

80-Hg-188