

Program EVALPLOT  
(Version 2021-1)

by

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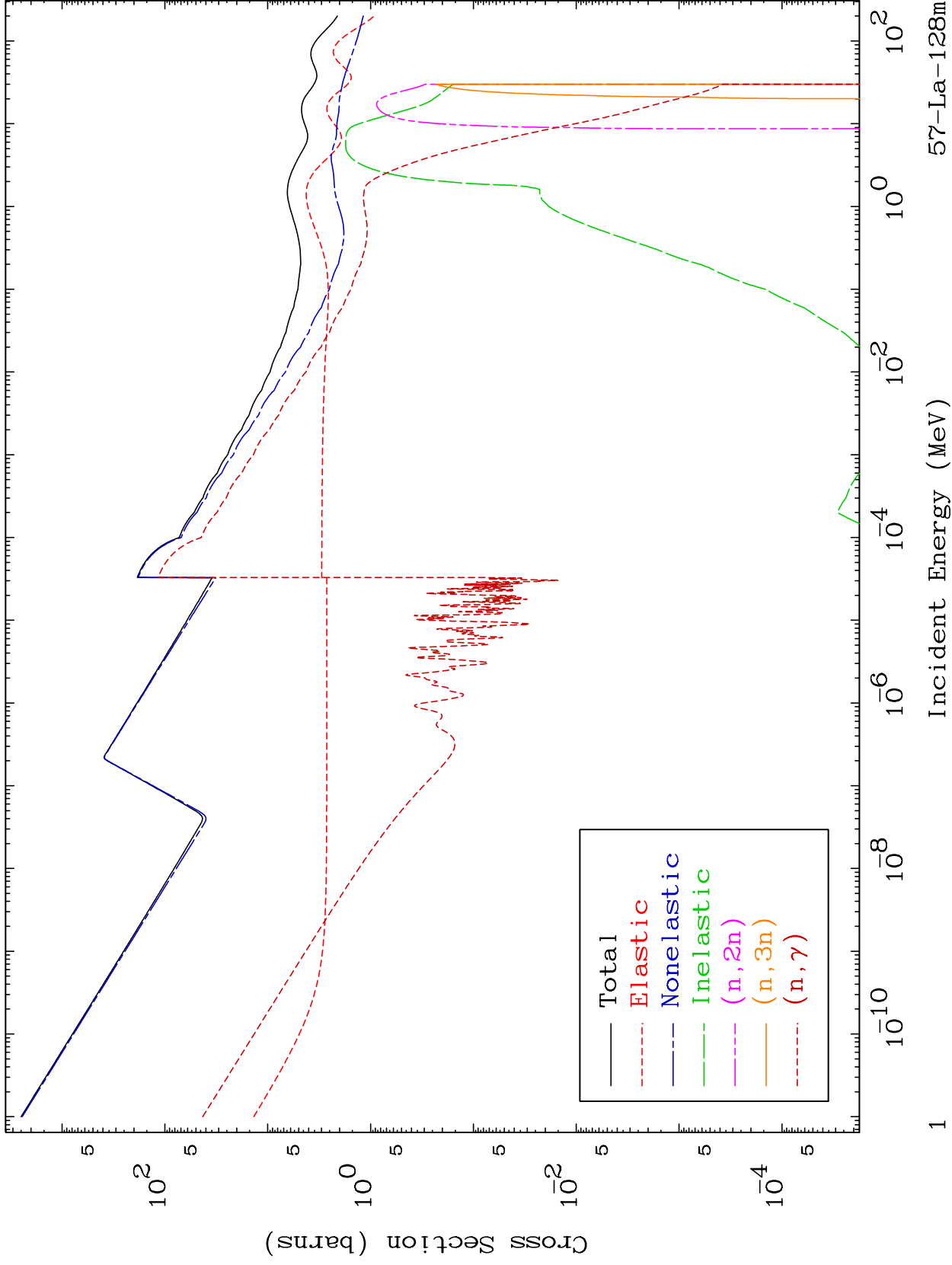
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 5696

Neutron Major  
293 Kelvin Cross Sections

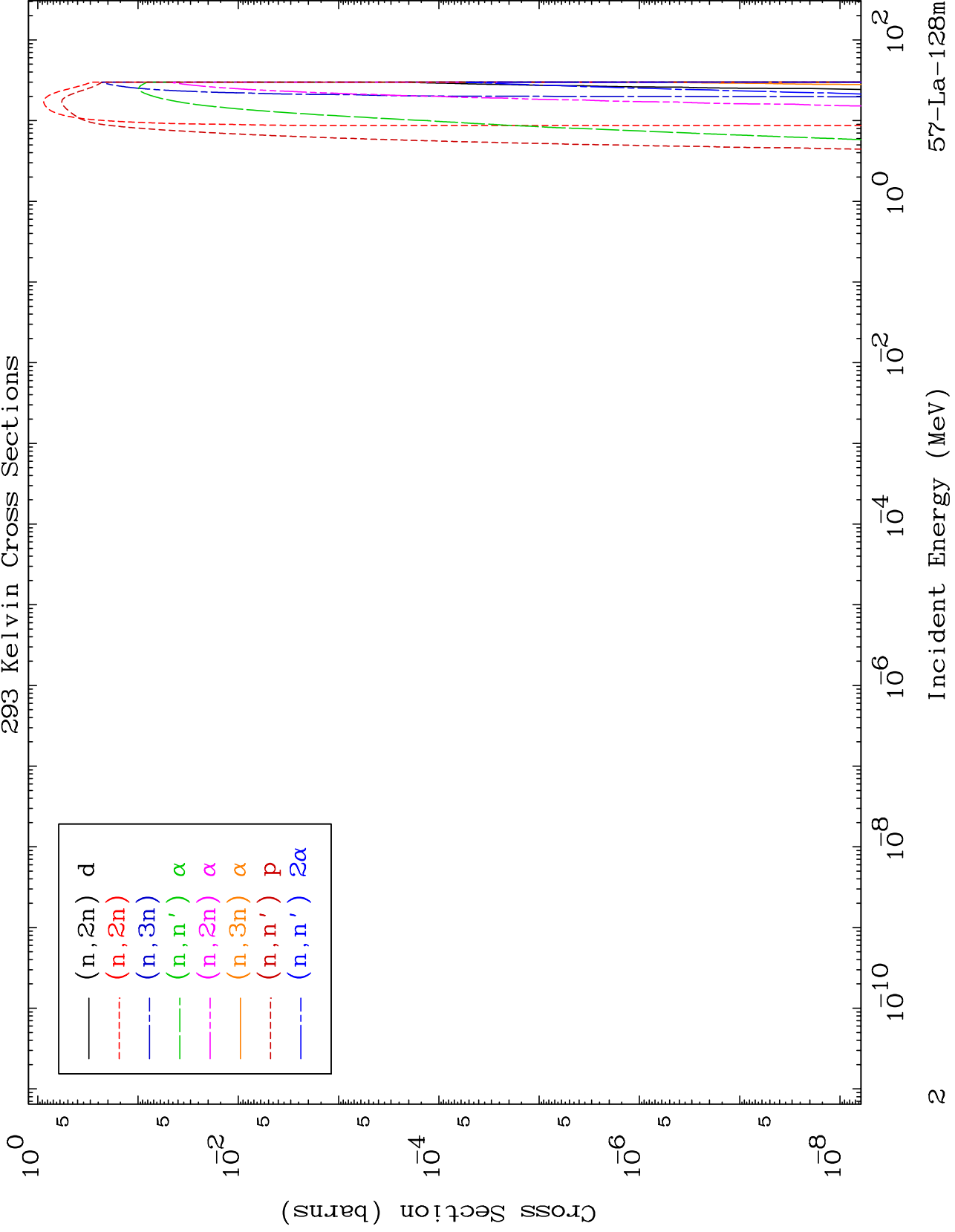
57-La-128m



MAT 5696

Neutron Absorption  
293 Kelvin Cross Sections

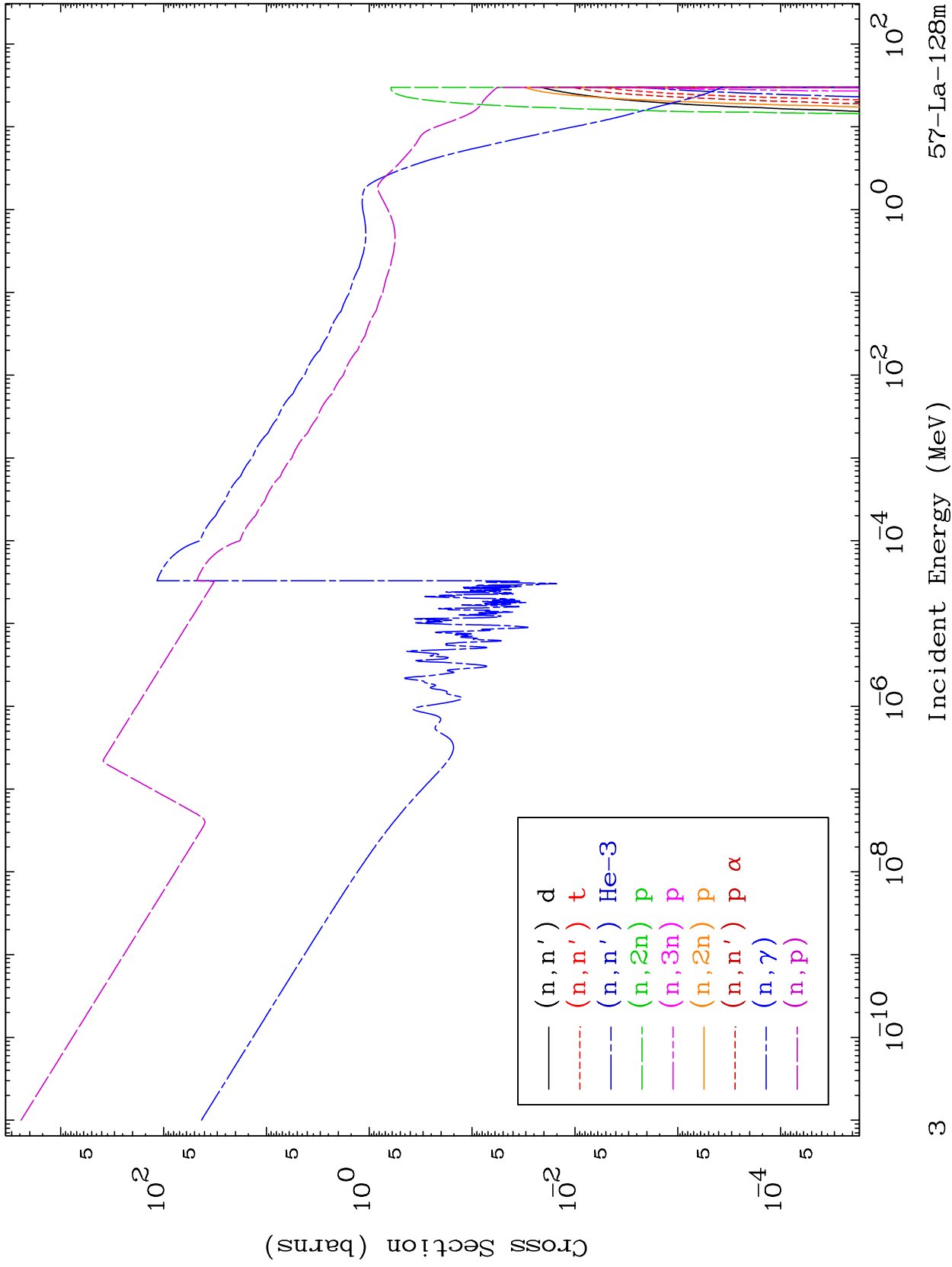
57-La-128m



MAT 5696

Neutron Absorption  
293 Kelvin Cross Sections

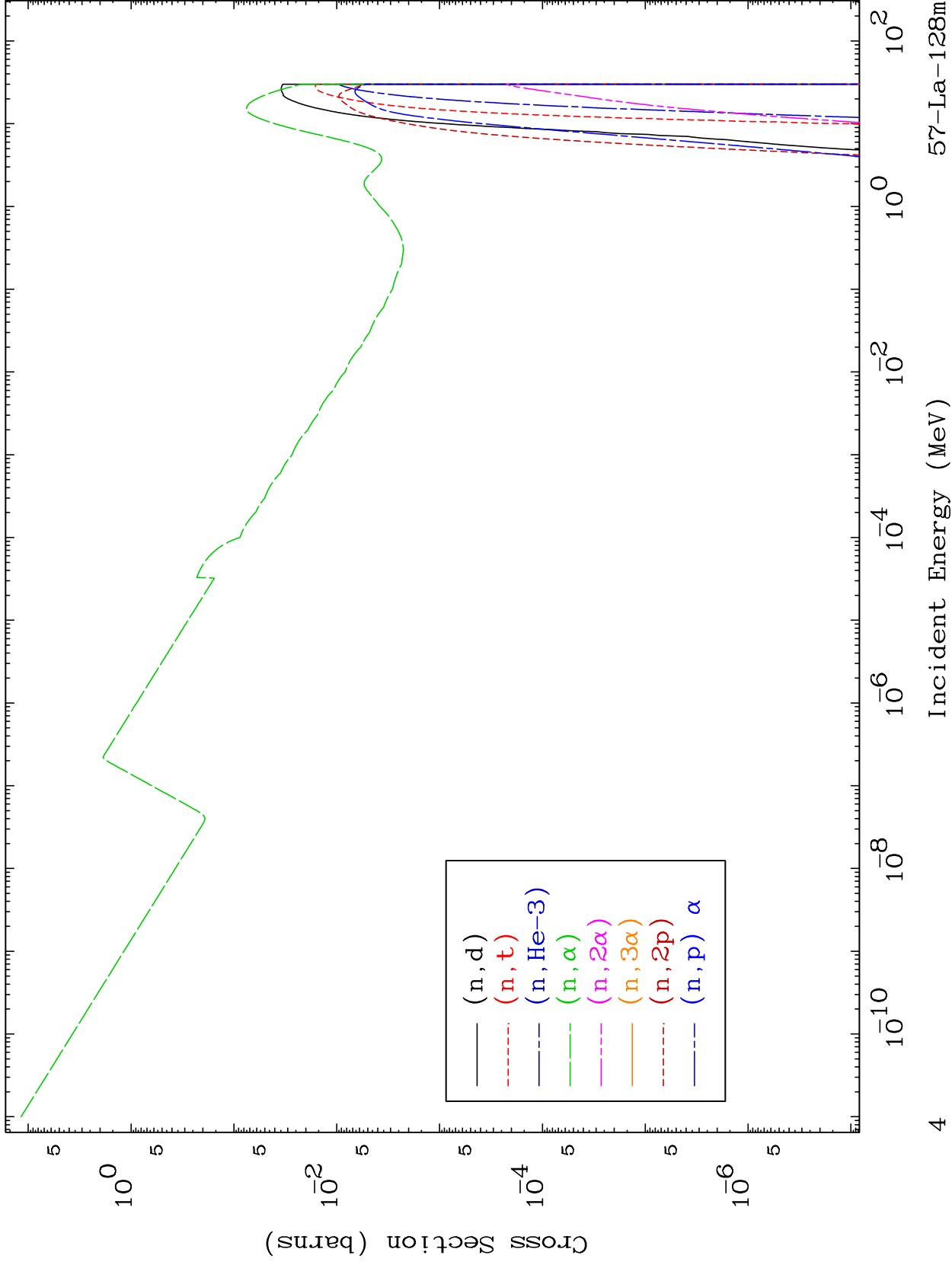
57-La-128m



MAT 5696

Neutron Absorption  
293 Kelvin Cross Sections

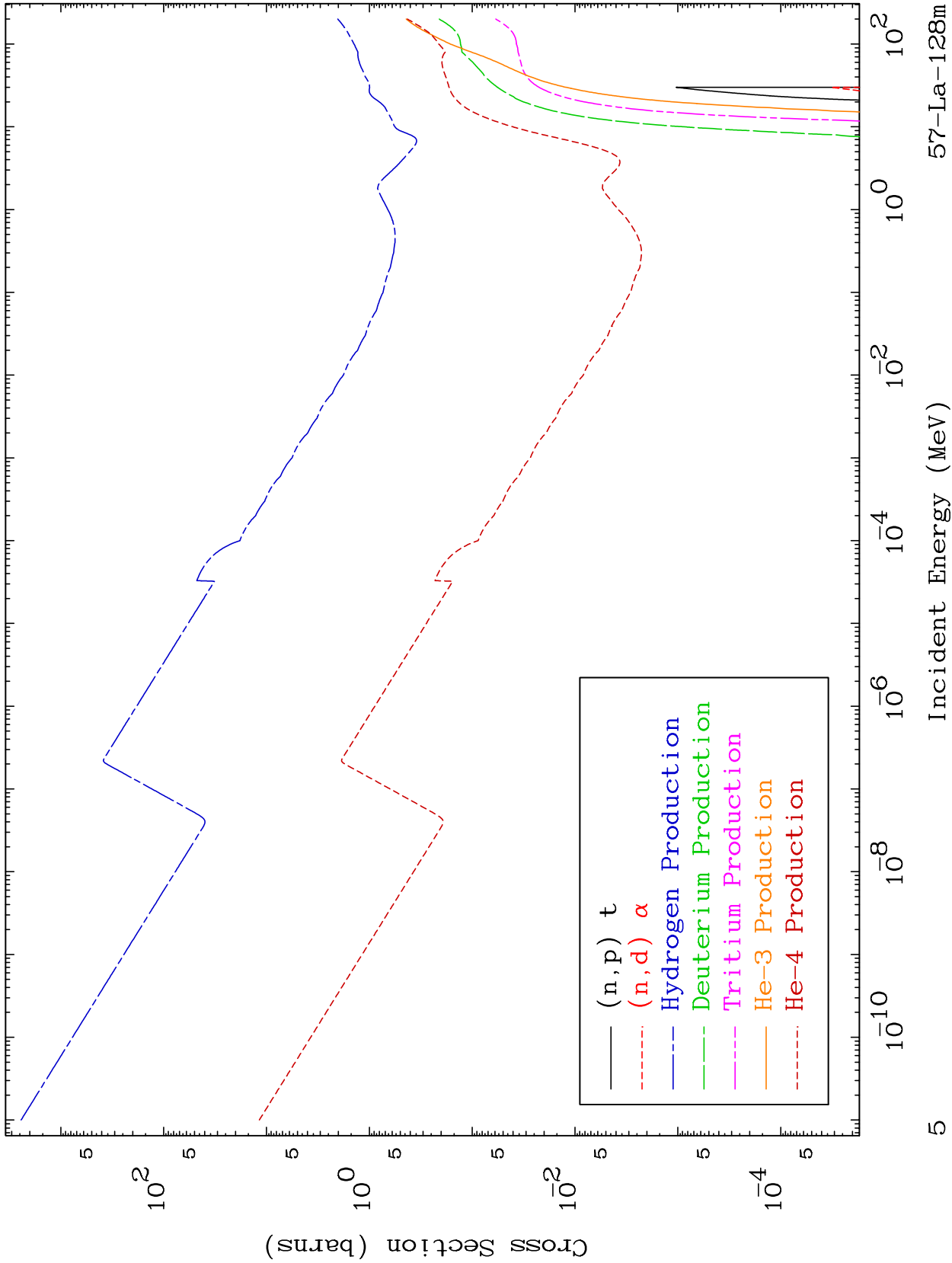
57-La-128m



MAT 5696

Neutron Absorption  
293 Kelvin Cross Sections

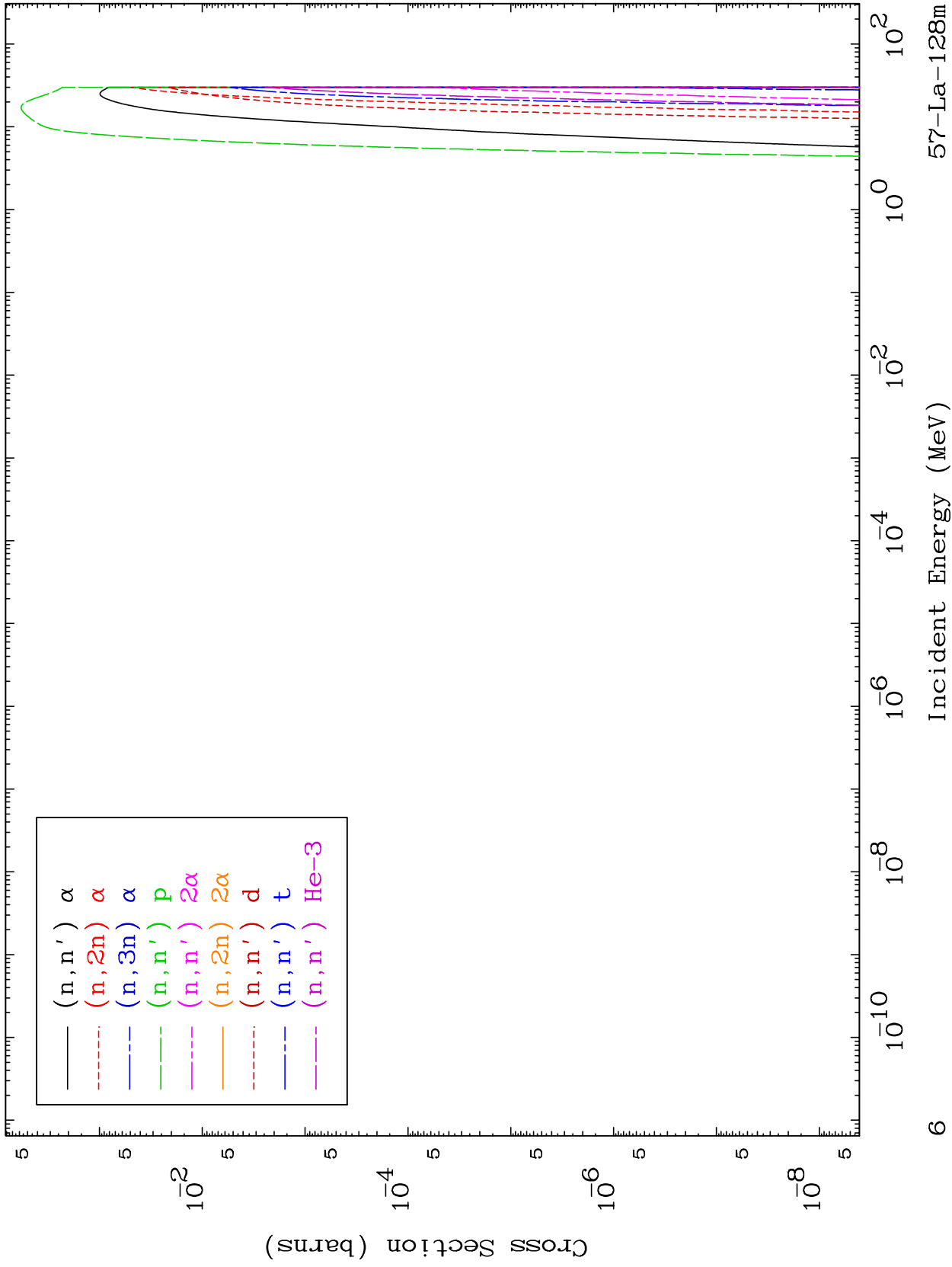
57-La-128m



MAT 5696

Charged Particle  
293 Kelvin Cross Sections

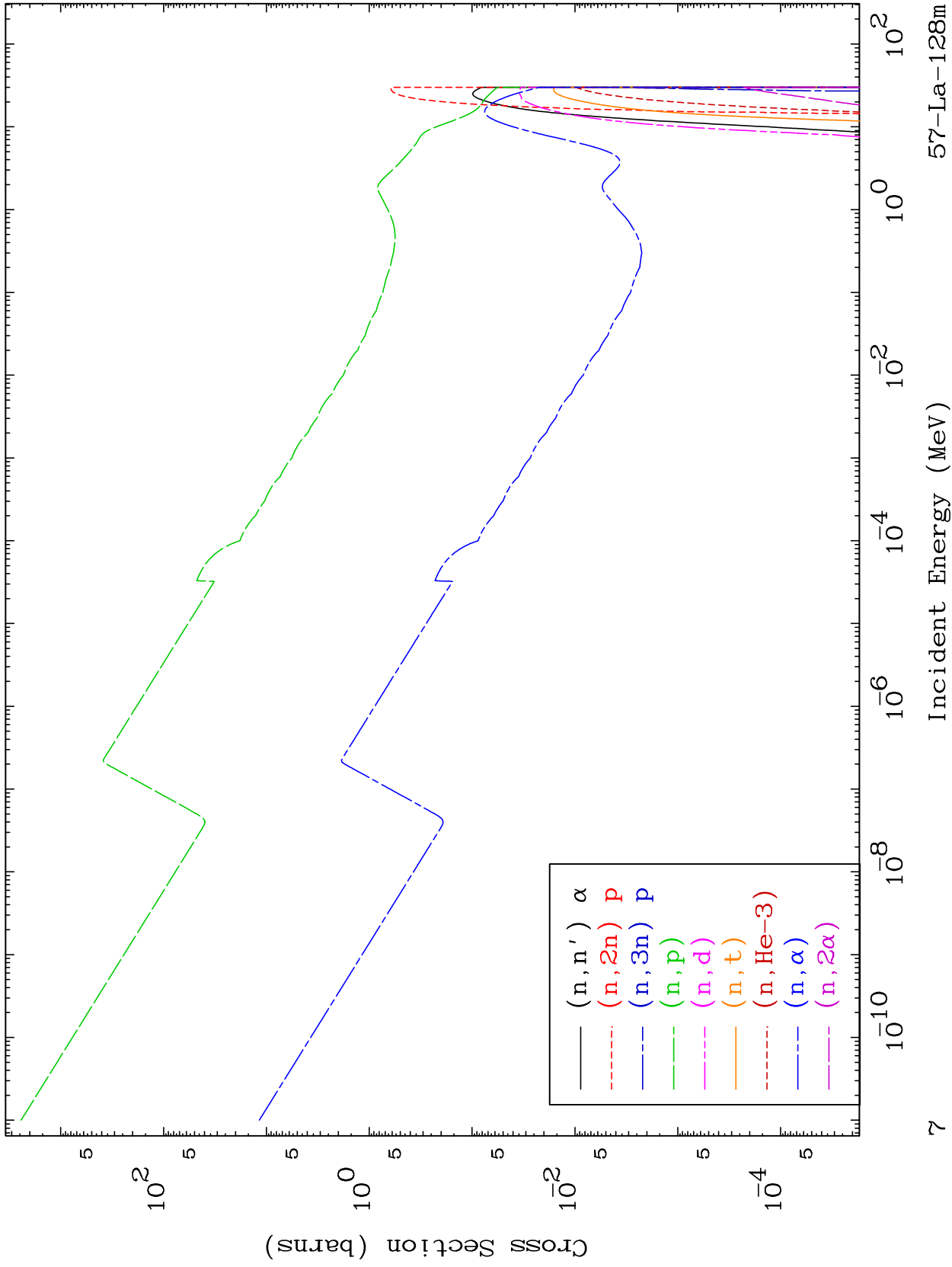
57-La-128m



MAT 5696

Charged Particle  
293 Kelvin Cross Sections

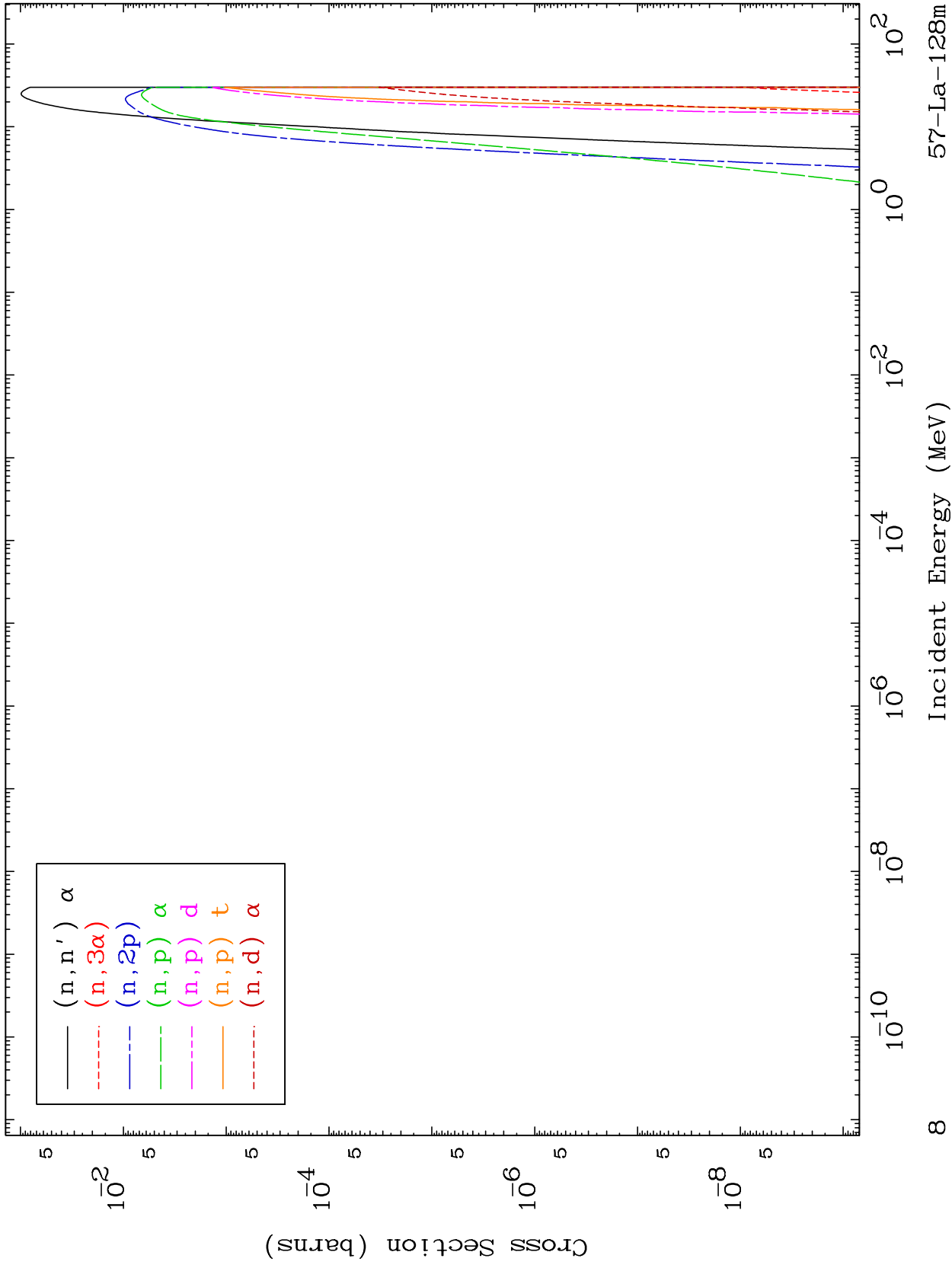
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Charged Particle  
293 Kelvin Cross Sections

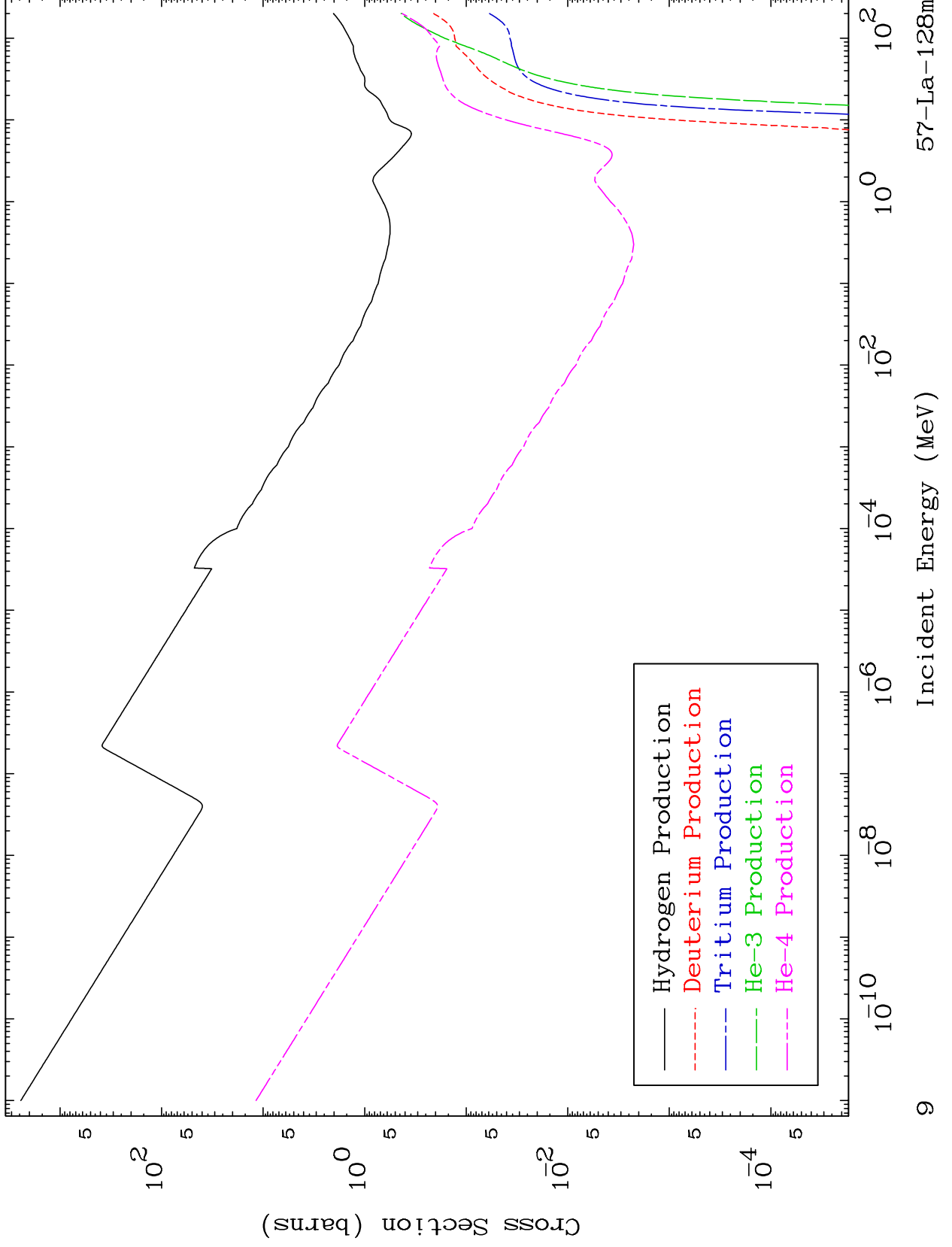
57-La-128m



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Particle Production  
293 Kelvin Cross Sections

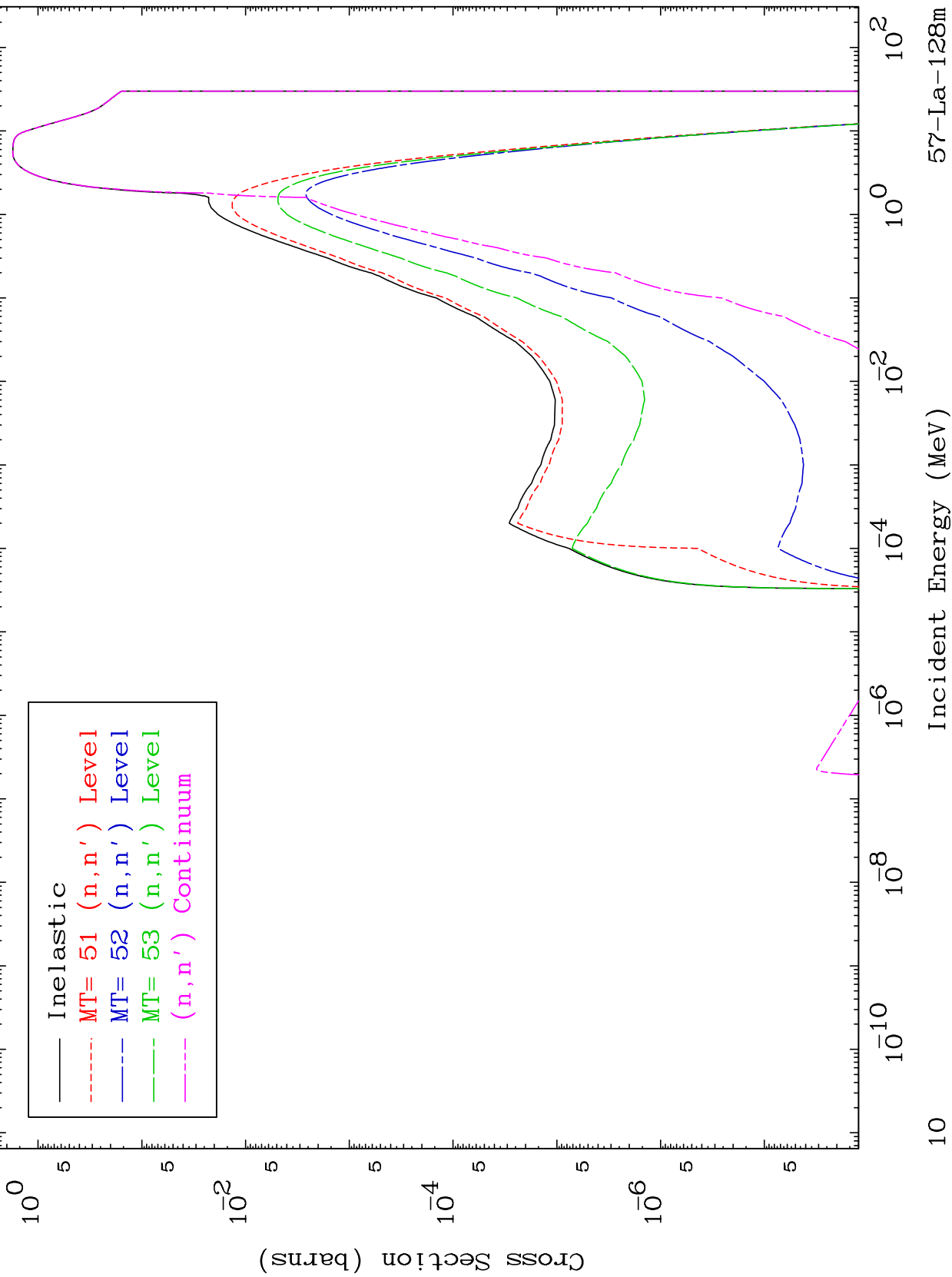
57-La-128m



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(n,n') Levels  
293 Kelvin Cross Sections

57-La-128m



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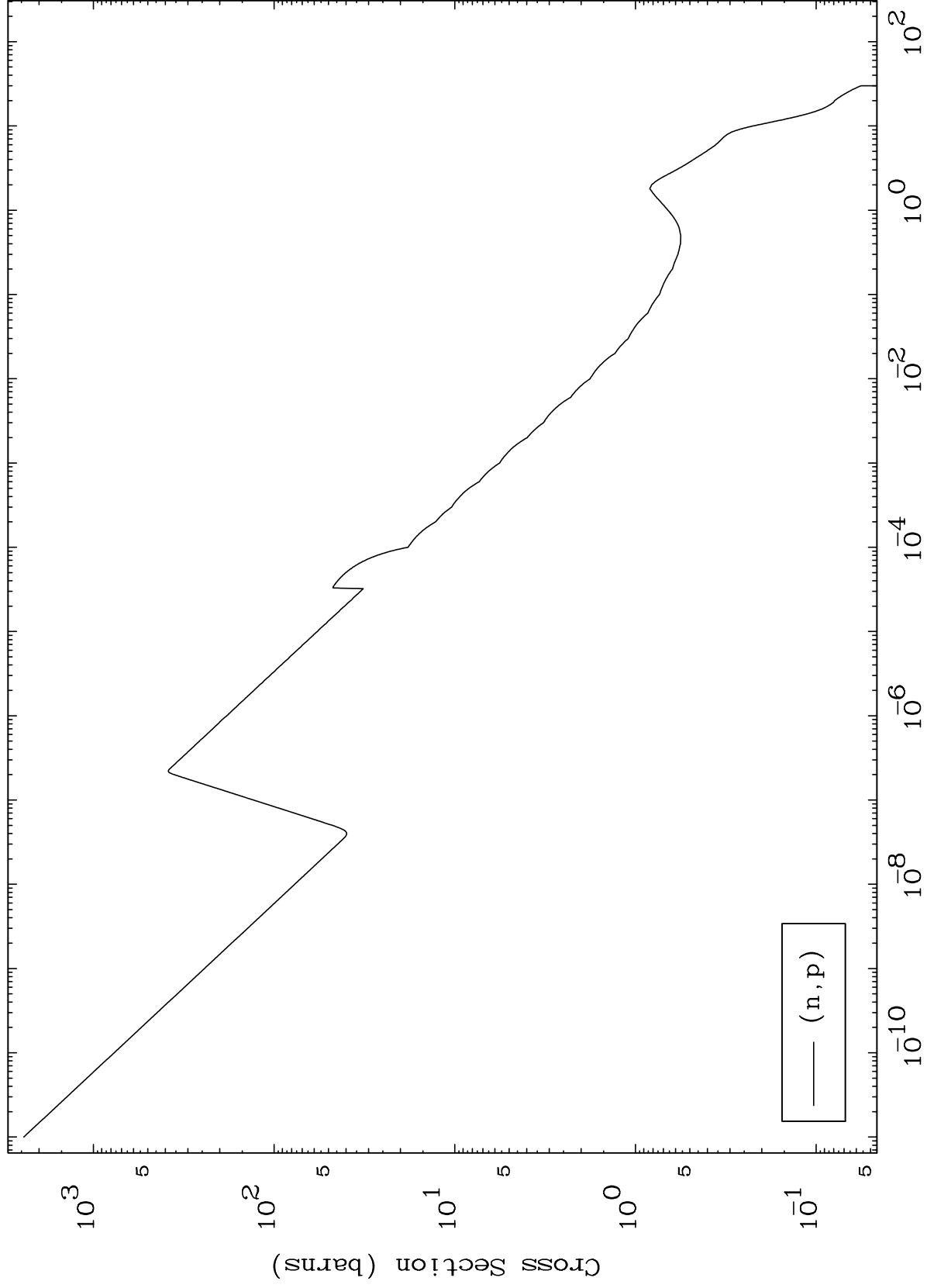
Incident Energy (MeV)

57-La-128m

MAT 5696

(n,p) Levels  
293 Kelvin Cross Sections

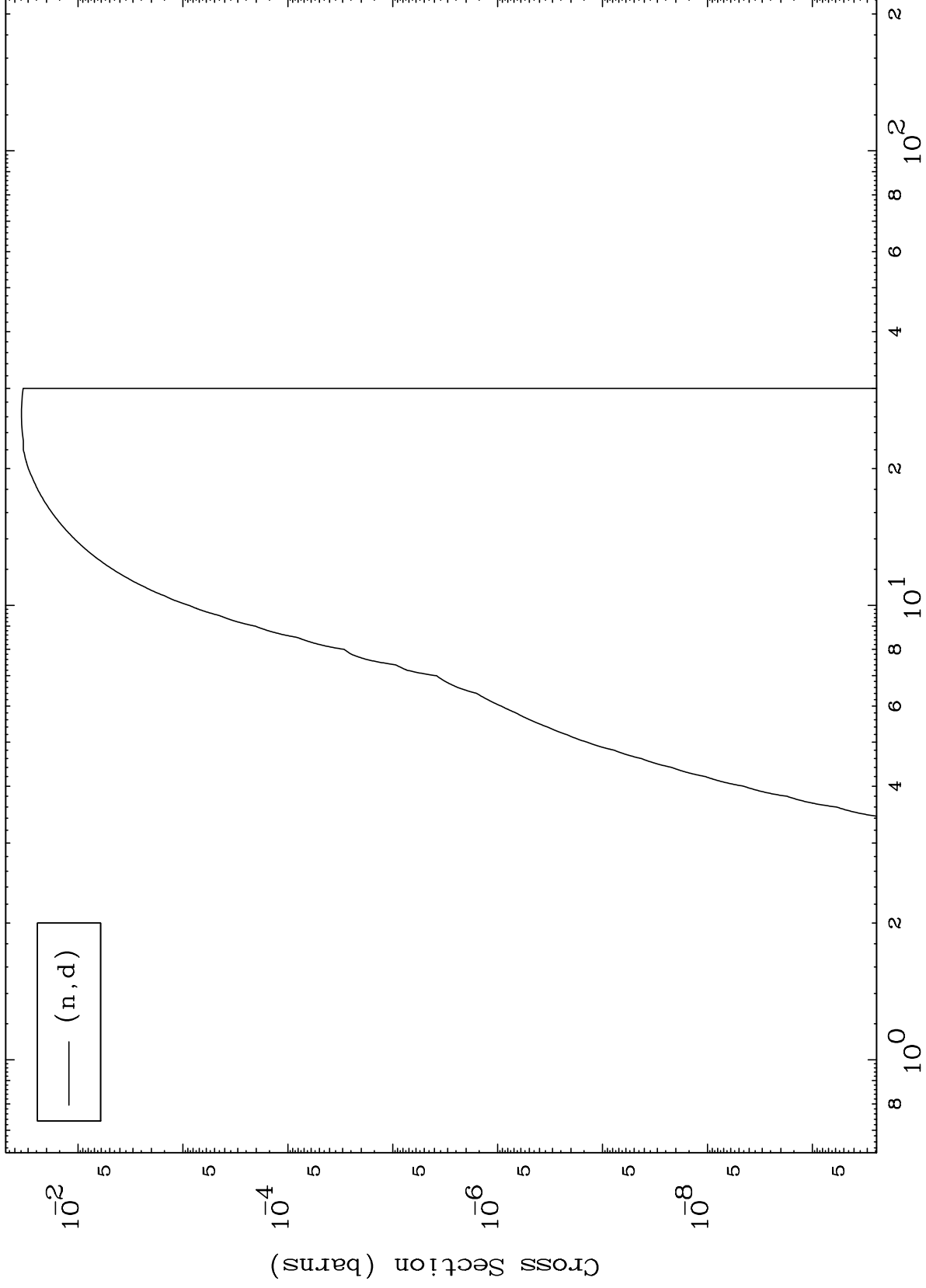
57-La-128m



MAT 5696

(n,d) Levels  
293 Kelvin Cross Sections

57-La-128m



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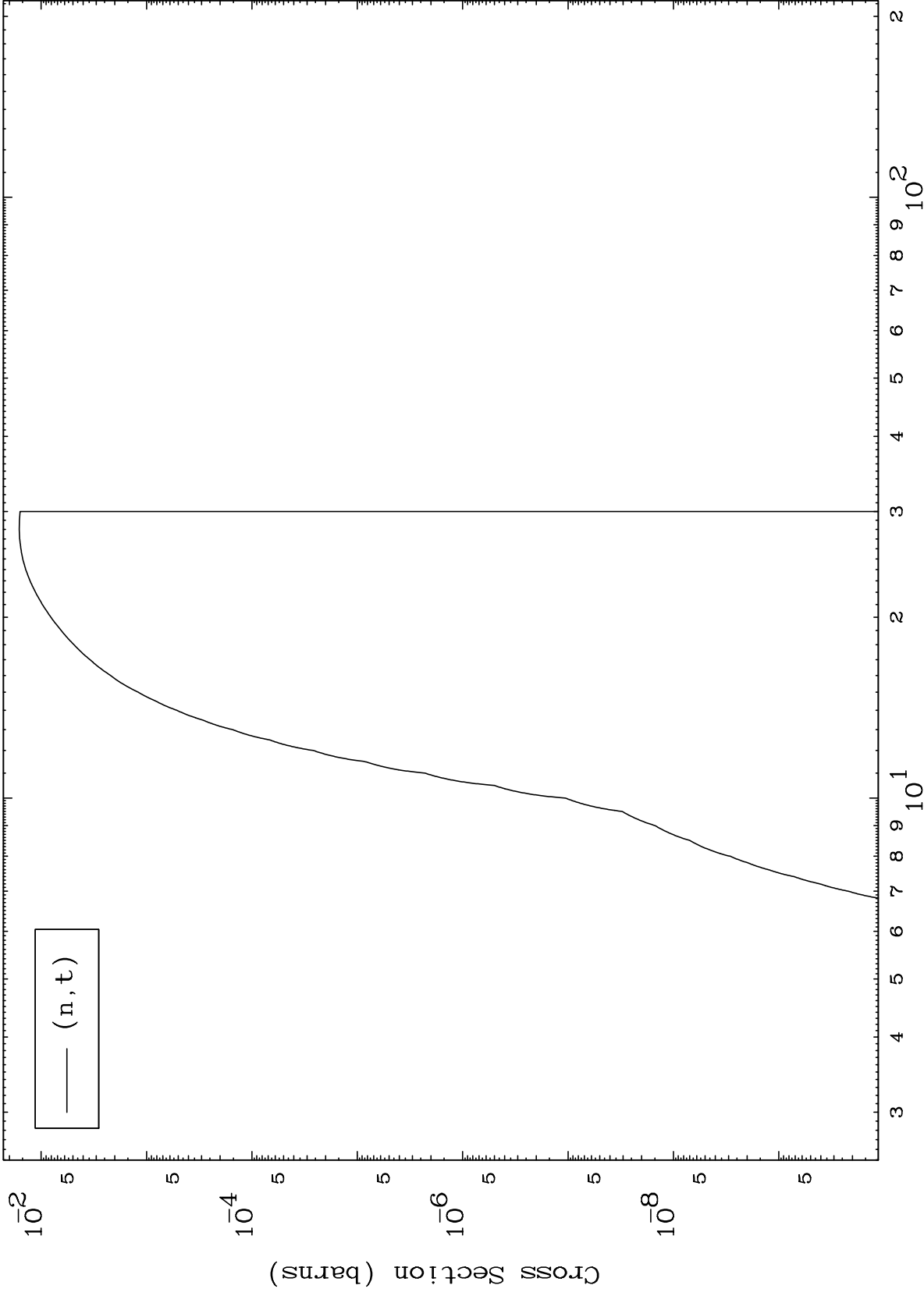
Incident Energy (MeV)

57-La-128m

MAT 5696

(n,t) Levels  
293 Kelvin Cross Sections

57-La-128m



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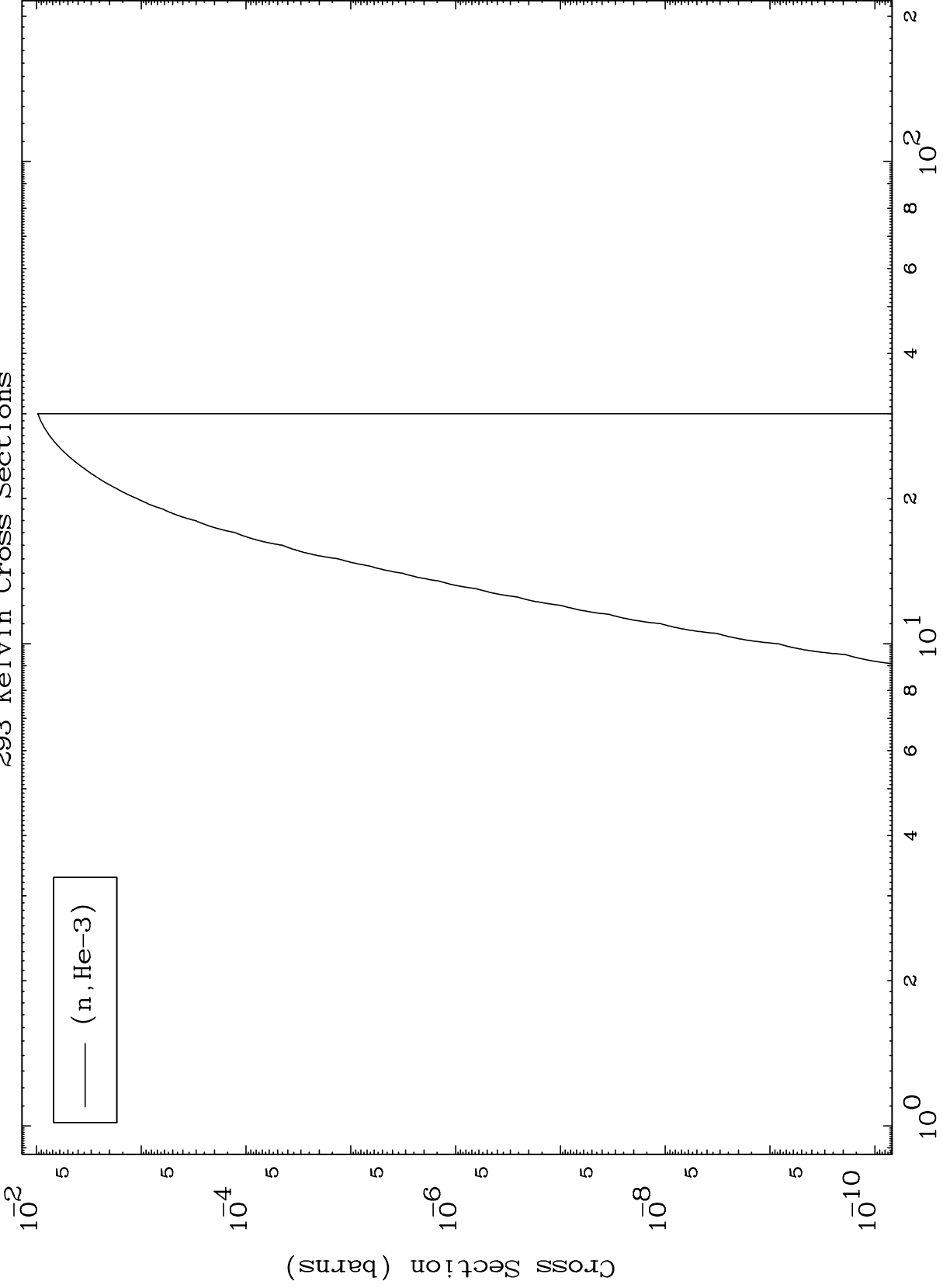
Incident Energy (MeV)

57-La-128m

MAT 5696

(n,He3) Levels  
293 Kelvin Cross Sections

57-La-128m



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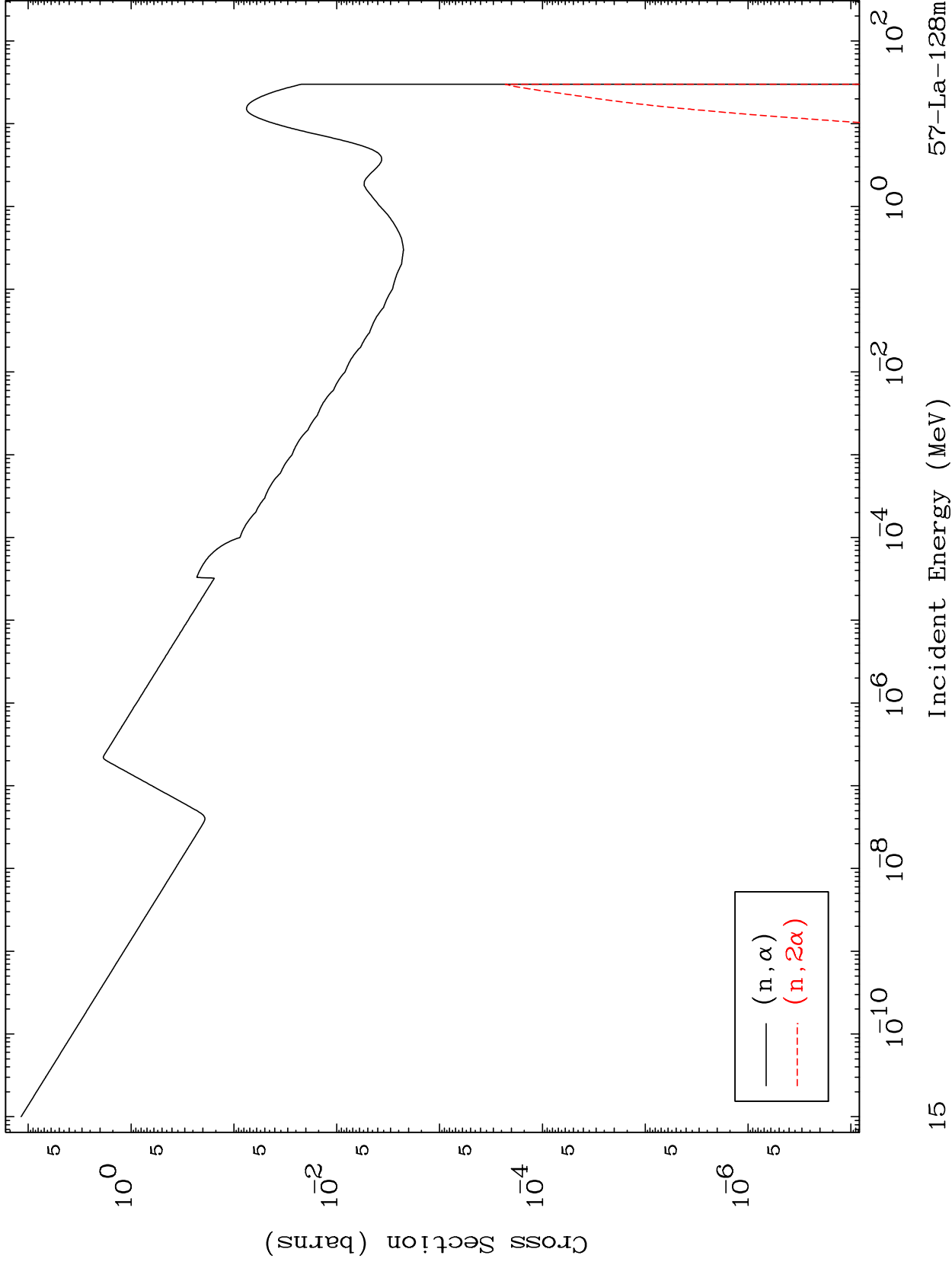
Incident Energy (MeV)

57-La-128m

MAT 5696

(n,α) Levels  
293 Kelvin Cross Sections

57-La-128m

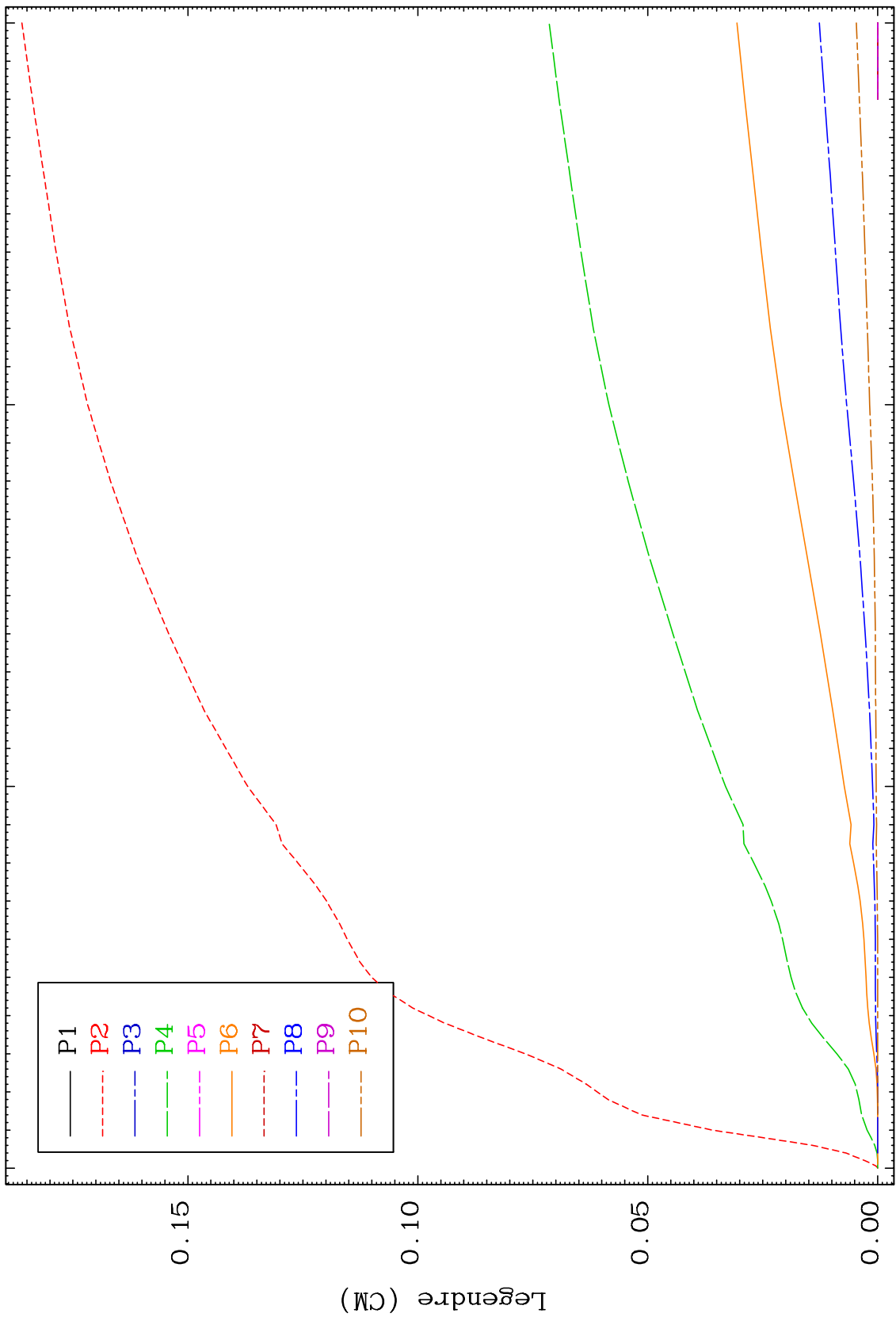


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Elastic Legendre Coefficients

57-La-128m



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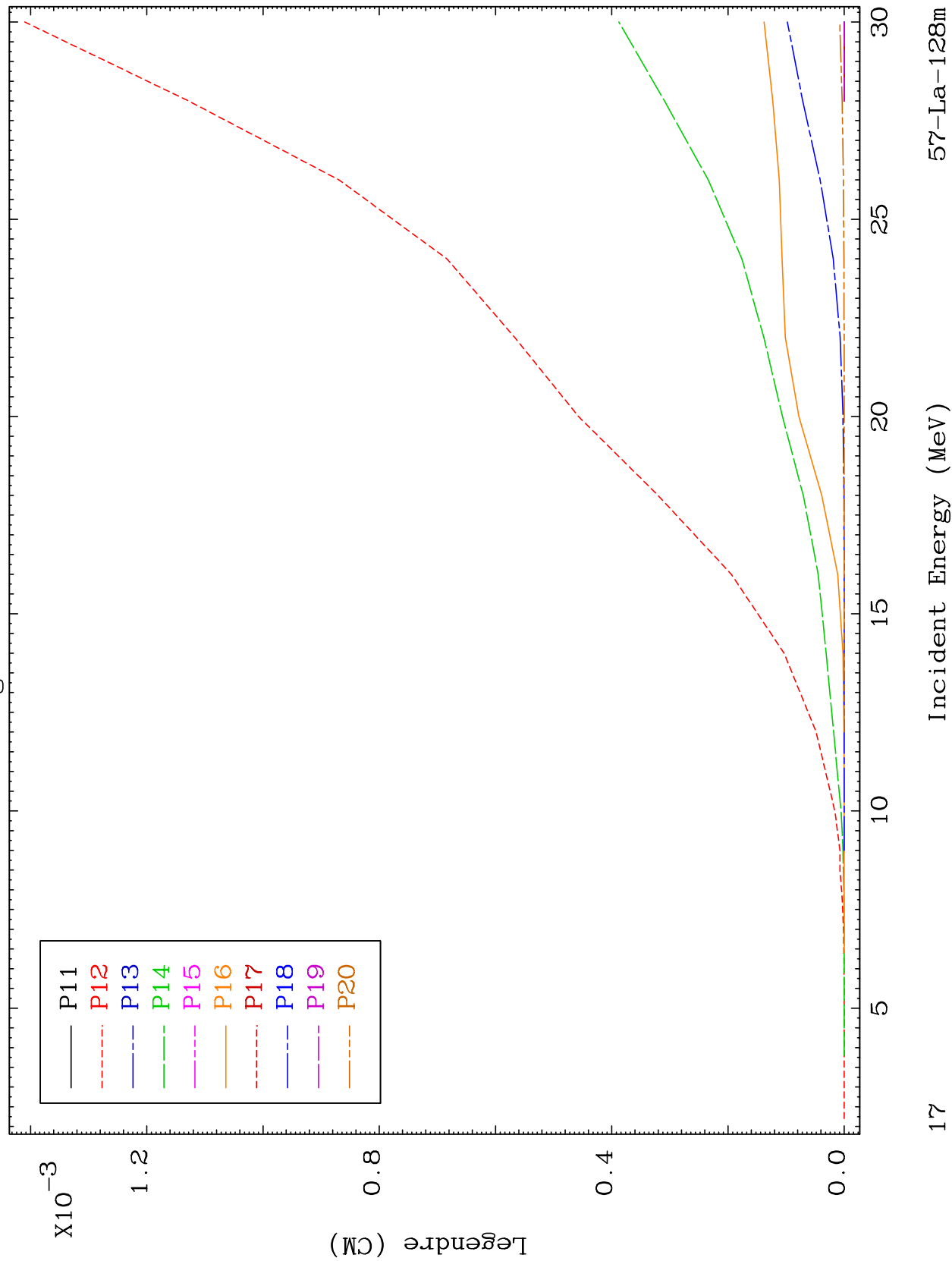
Incident Energy (MeV)

57-La-128m

MAT 5696

Elastic Legendre Coefficients

57-La-128m



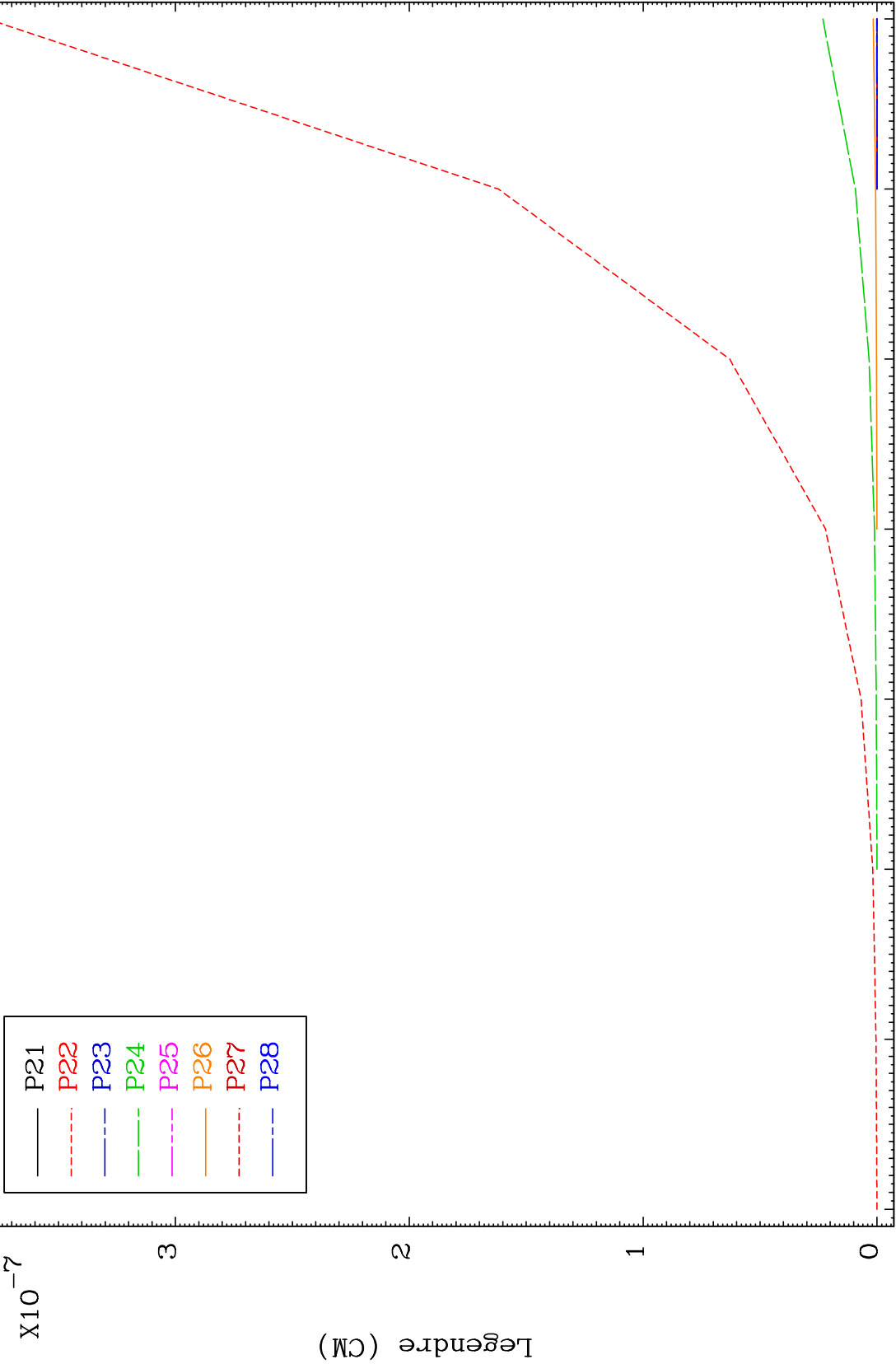
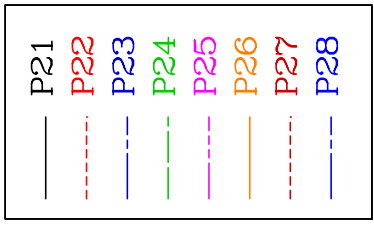
17

57-La-128m

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Elastic Legendre Coefficients

57-La-128m



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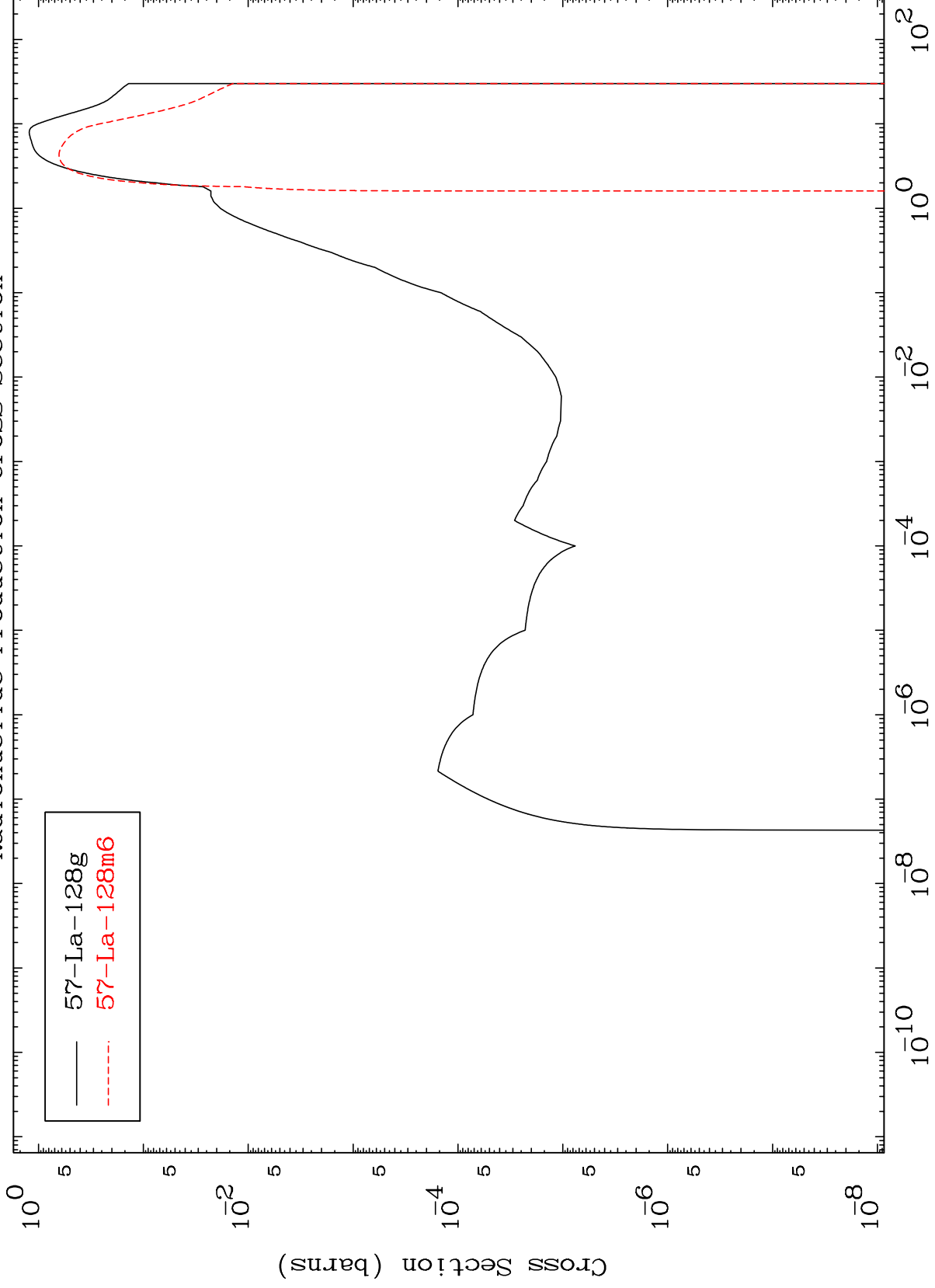
Incident Energy (MeV)

57-La-128m

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Inelastic  
Radionuclide Production Cross Section

57-La-128m

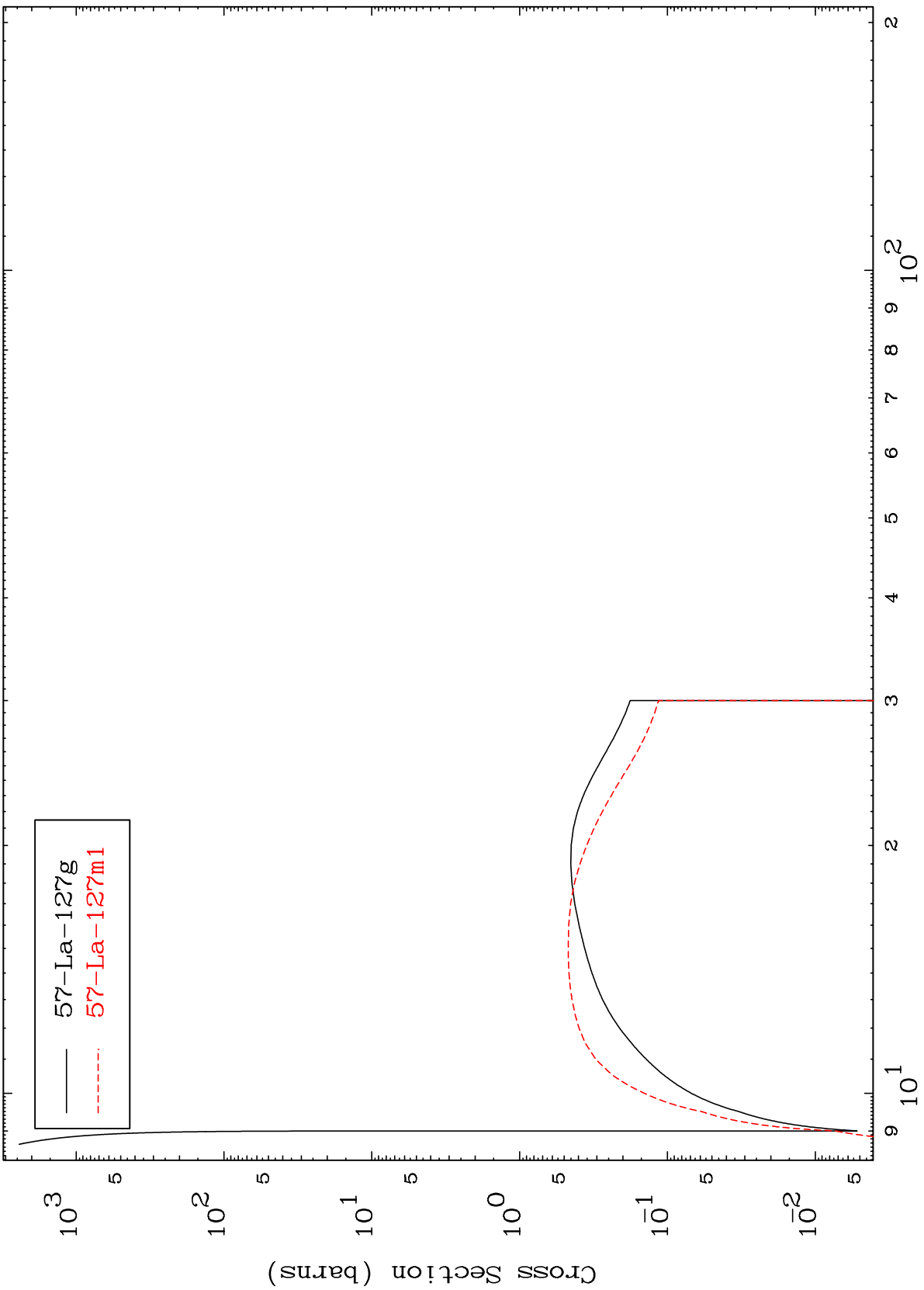


57-La-128m

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57-La-128m

Radionuclide Production Cross Section



57-La-128m

Incident Energy (MeV)

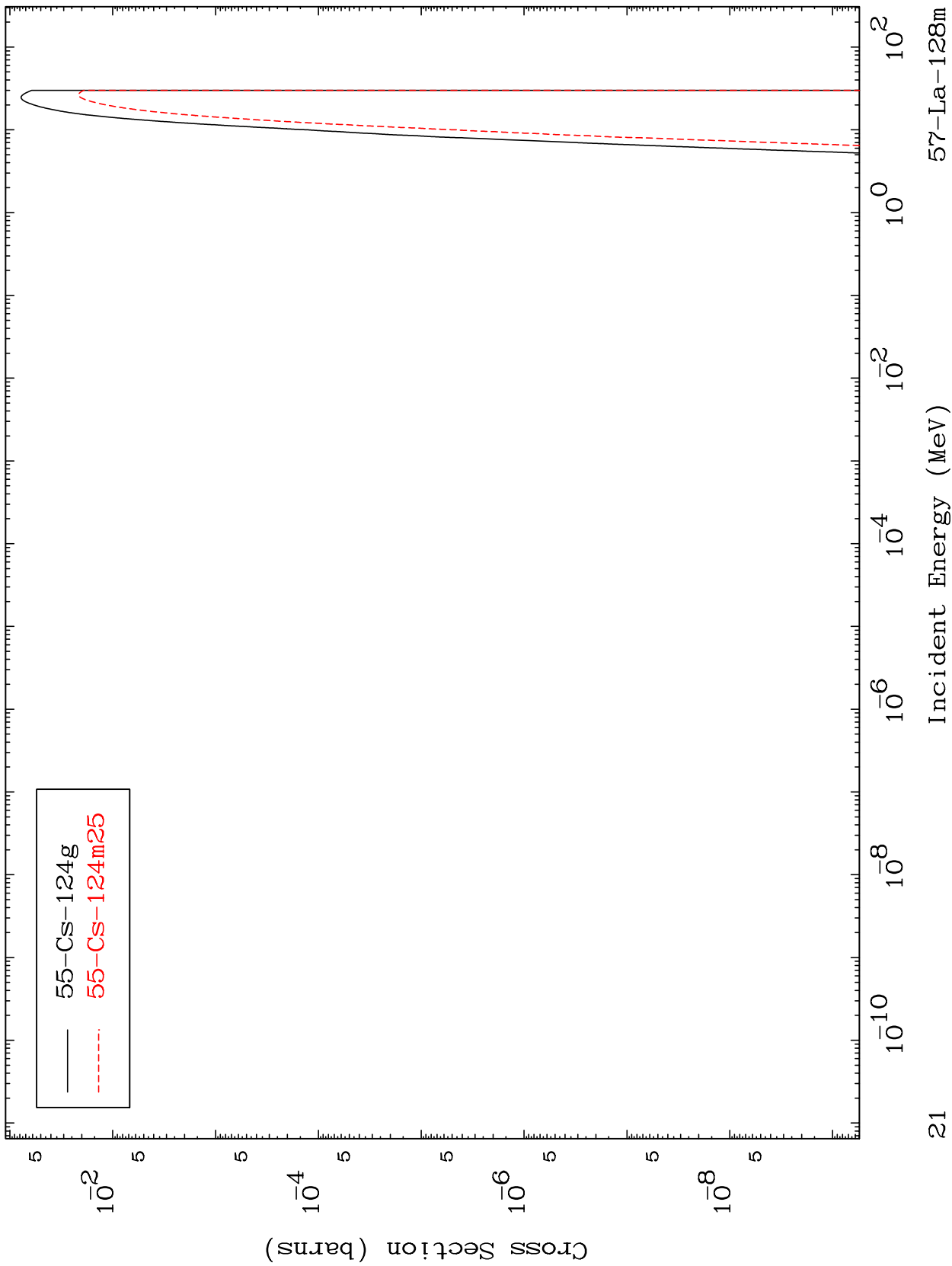
20

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$(n, n') \alpha$

57-La-128m

Radionuclide Production Cross Section

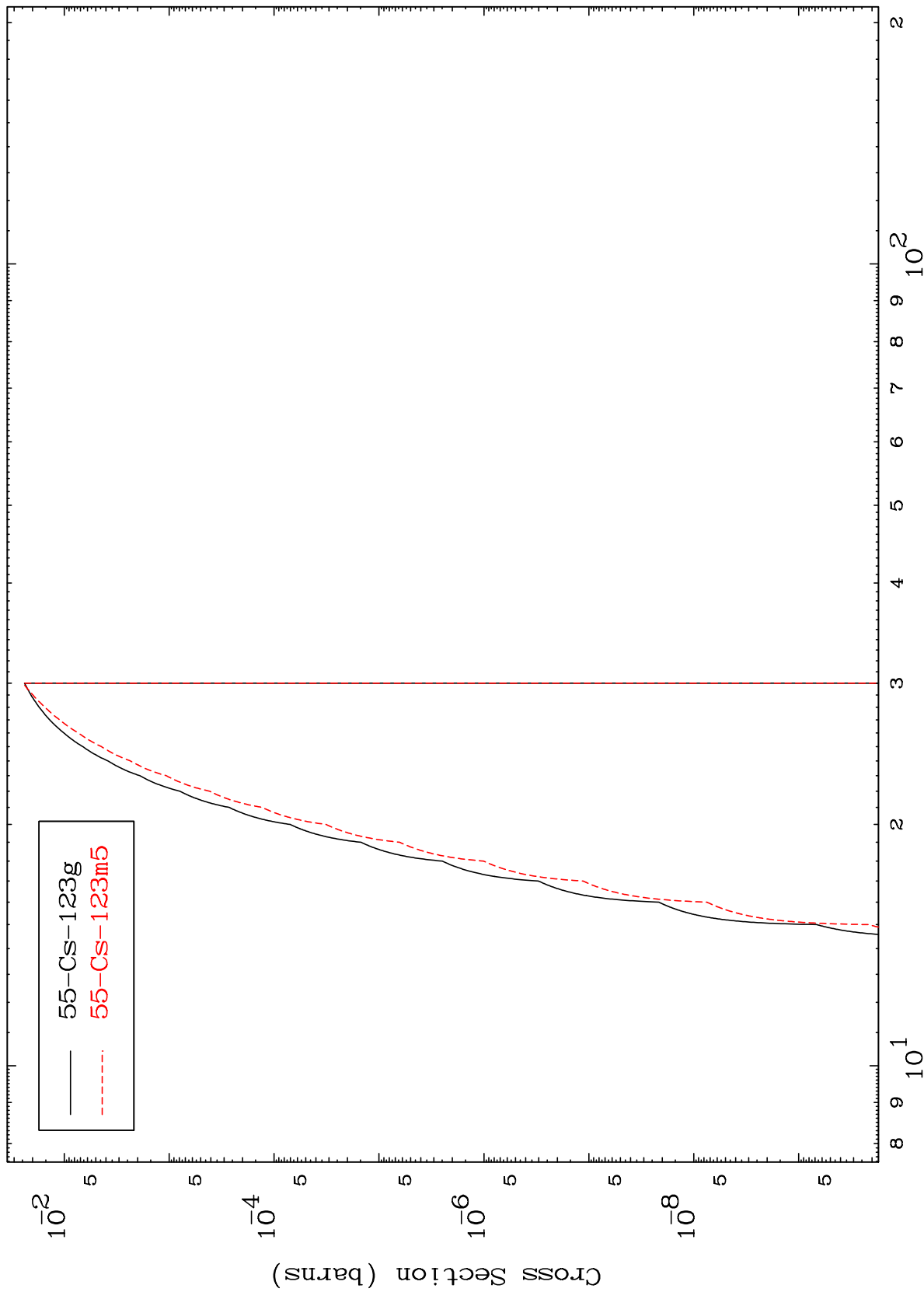


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(n,2n)  $\alpha$

57-La-128m

Radionuclide Production Cross Section

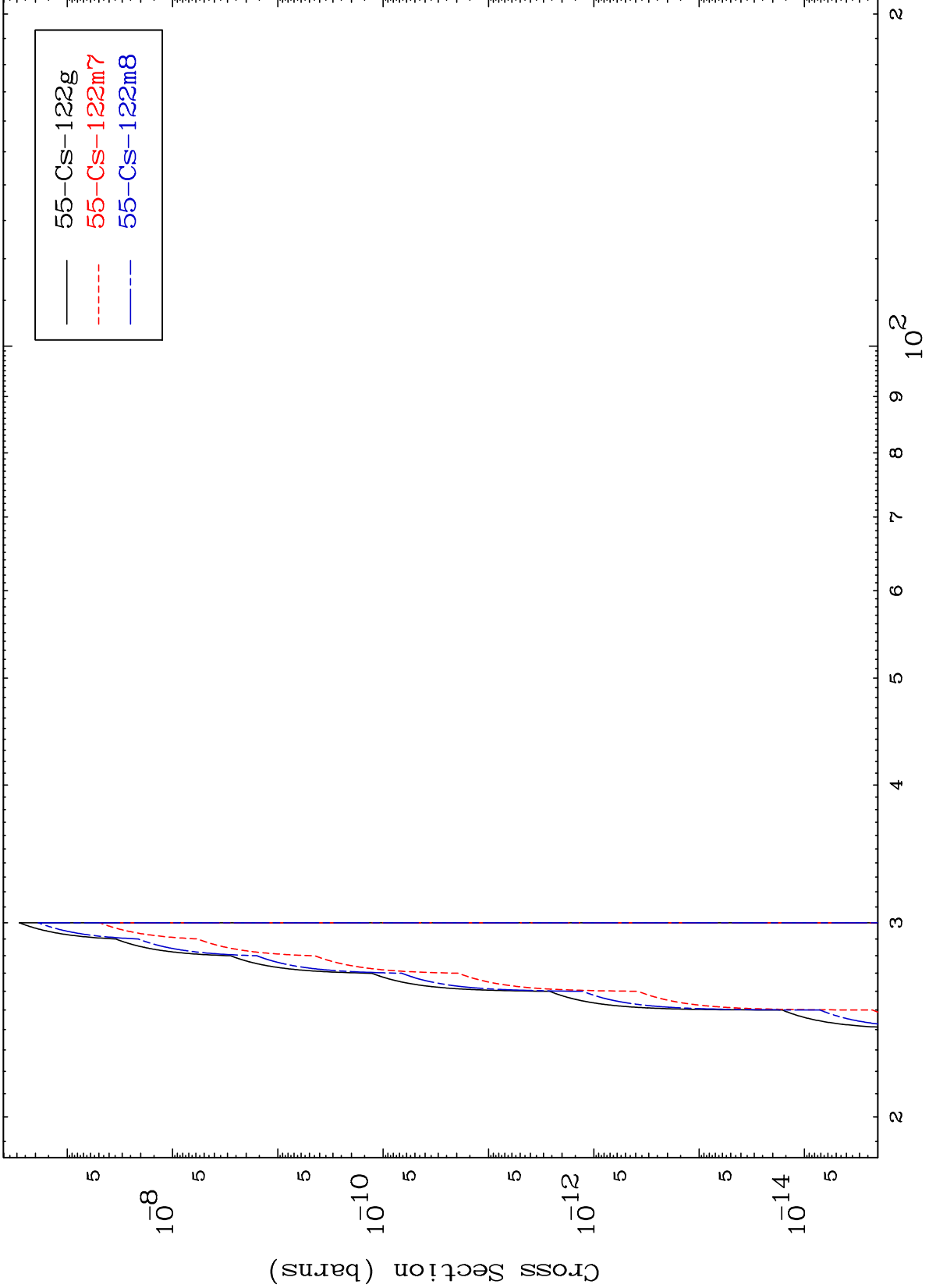


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(n,3n)  $\alpha$

57-La-128m

Radionuclide Production Cross Section



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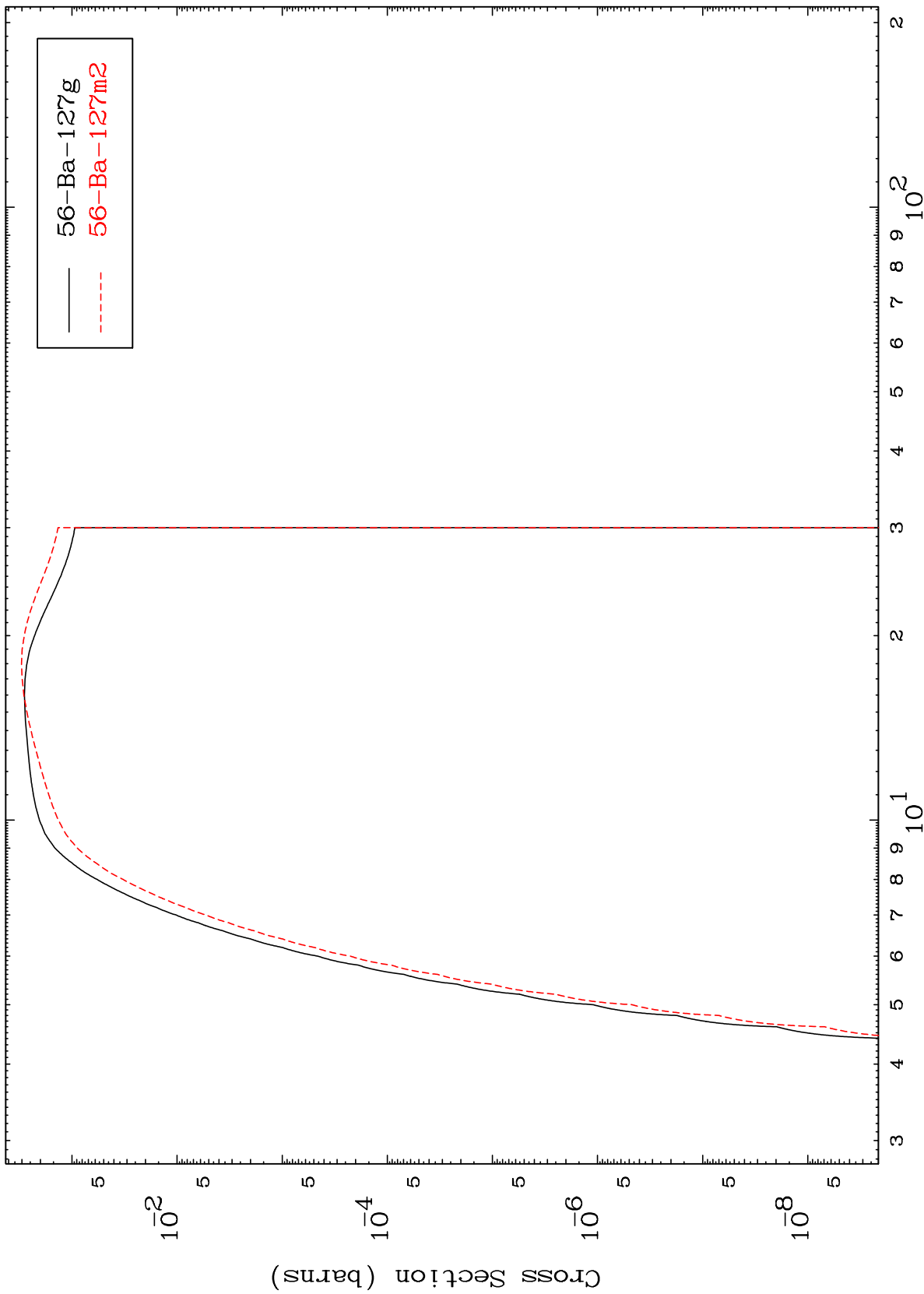
Incident Energy (MeV)

57-La-128m

MAT 5696

57-La-128m

$(n, n')$  p  
Radionuclide Production Cross Section



24

Incident Energy (MeV)

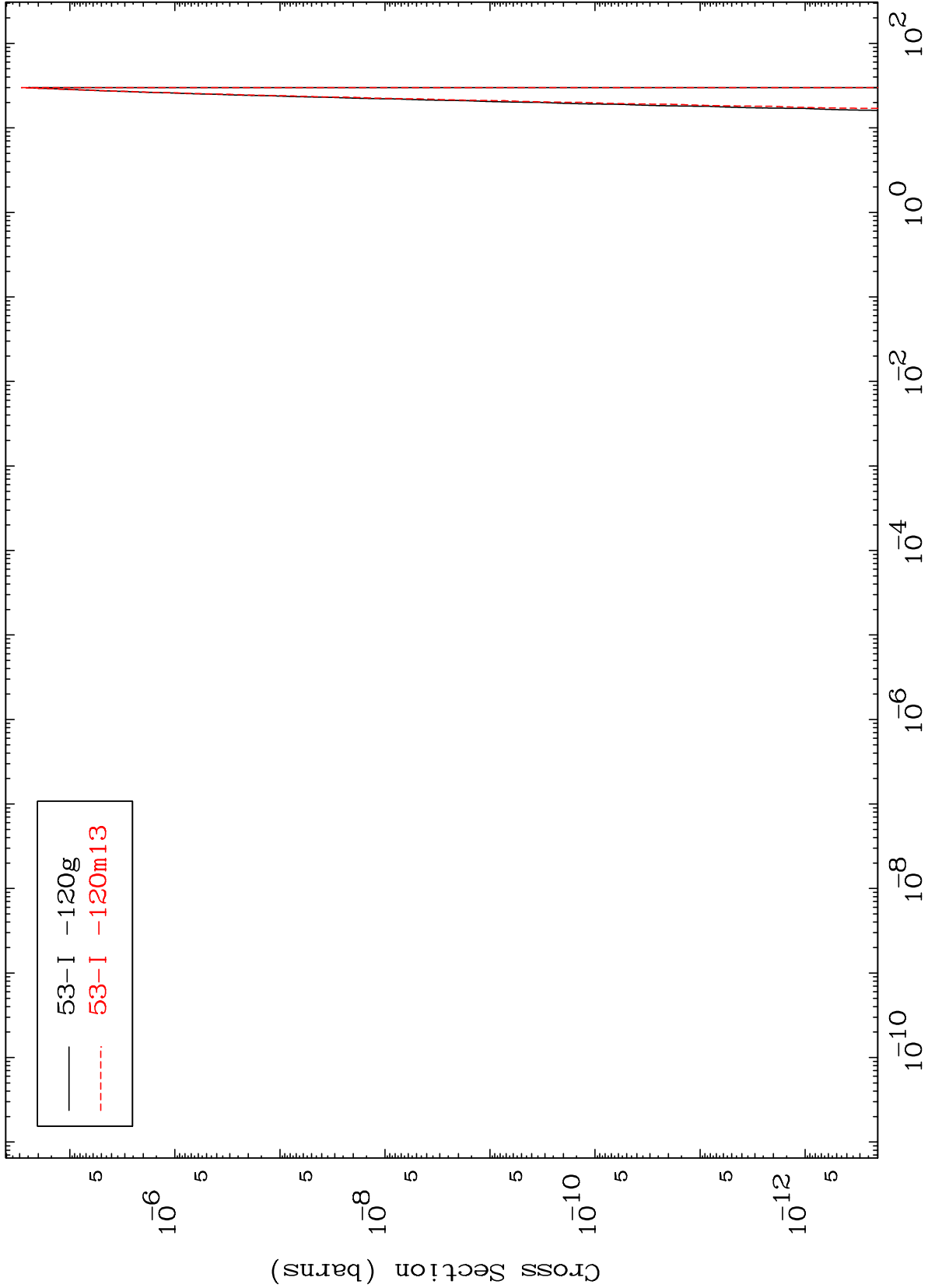
57-La-128m

MAT 5696

(n,n') 2 $\alpha$

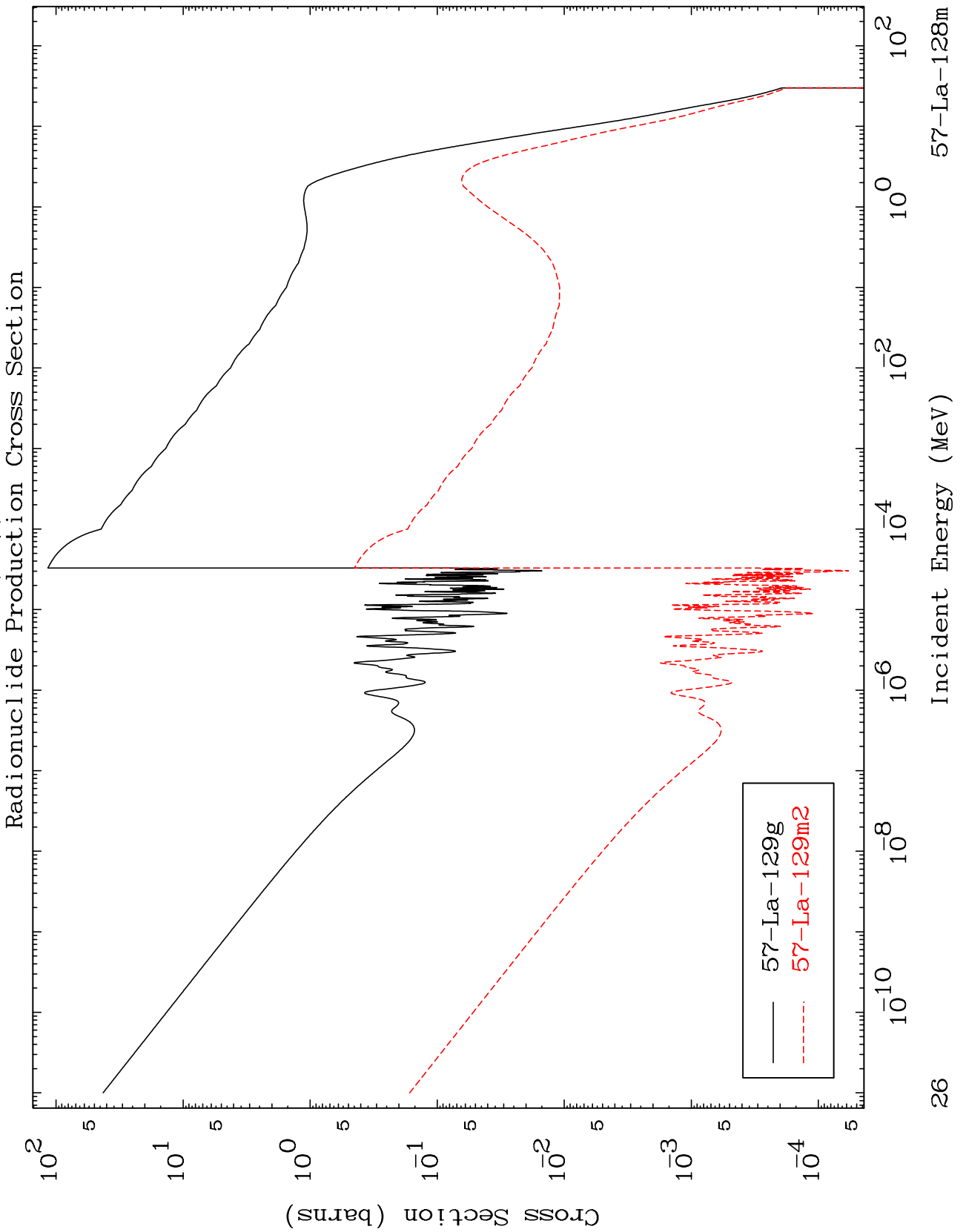
57-La-128m

Radionuclide Production Cross Section



MAT 5696

57-La-128m

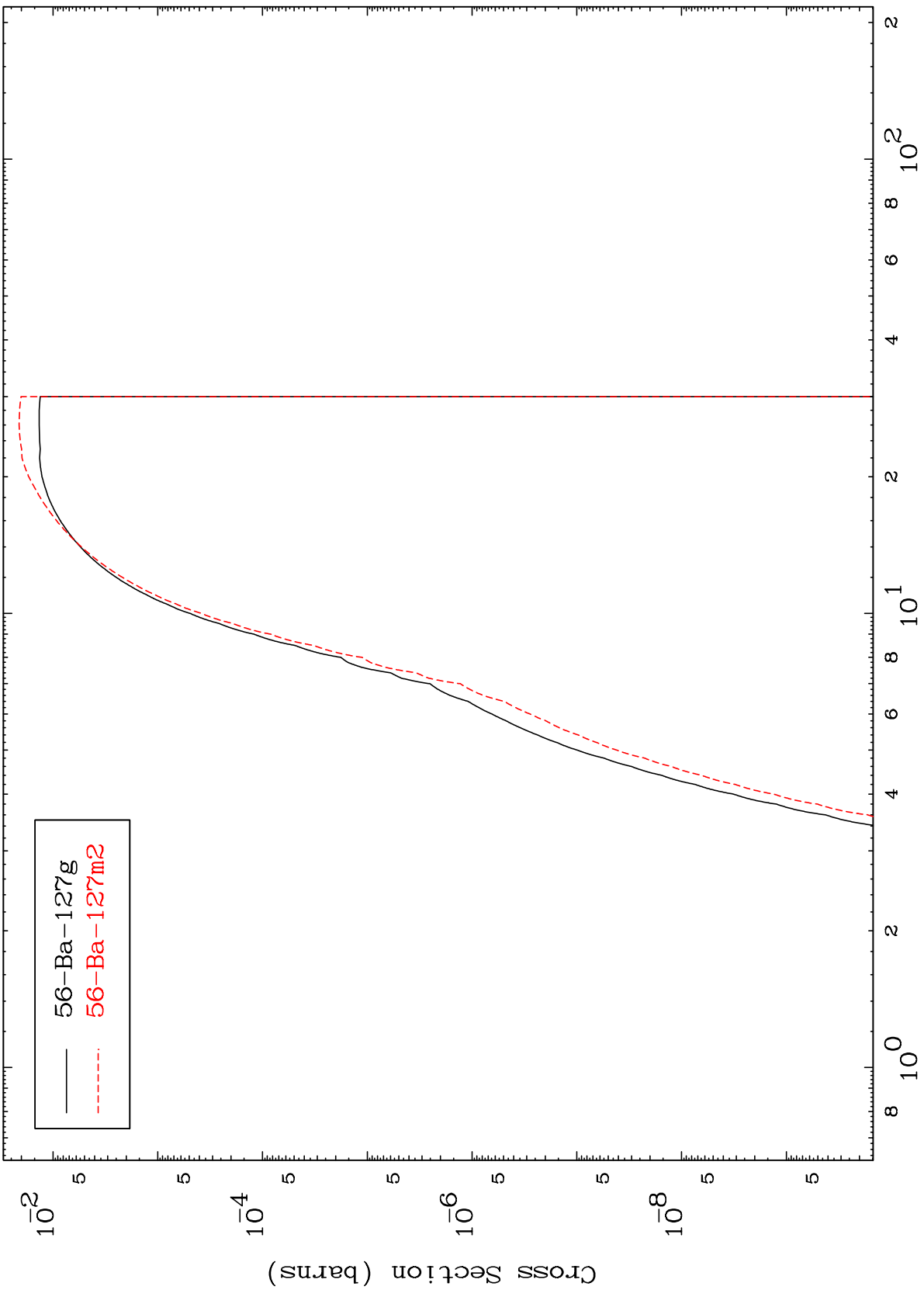


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(n,d)

57-La-128m

Radionuclide Production Cross Section



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Incident Energy (MeV)

57-La-128m