

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

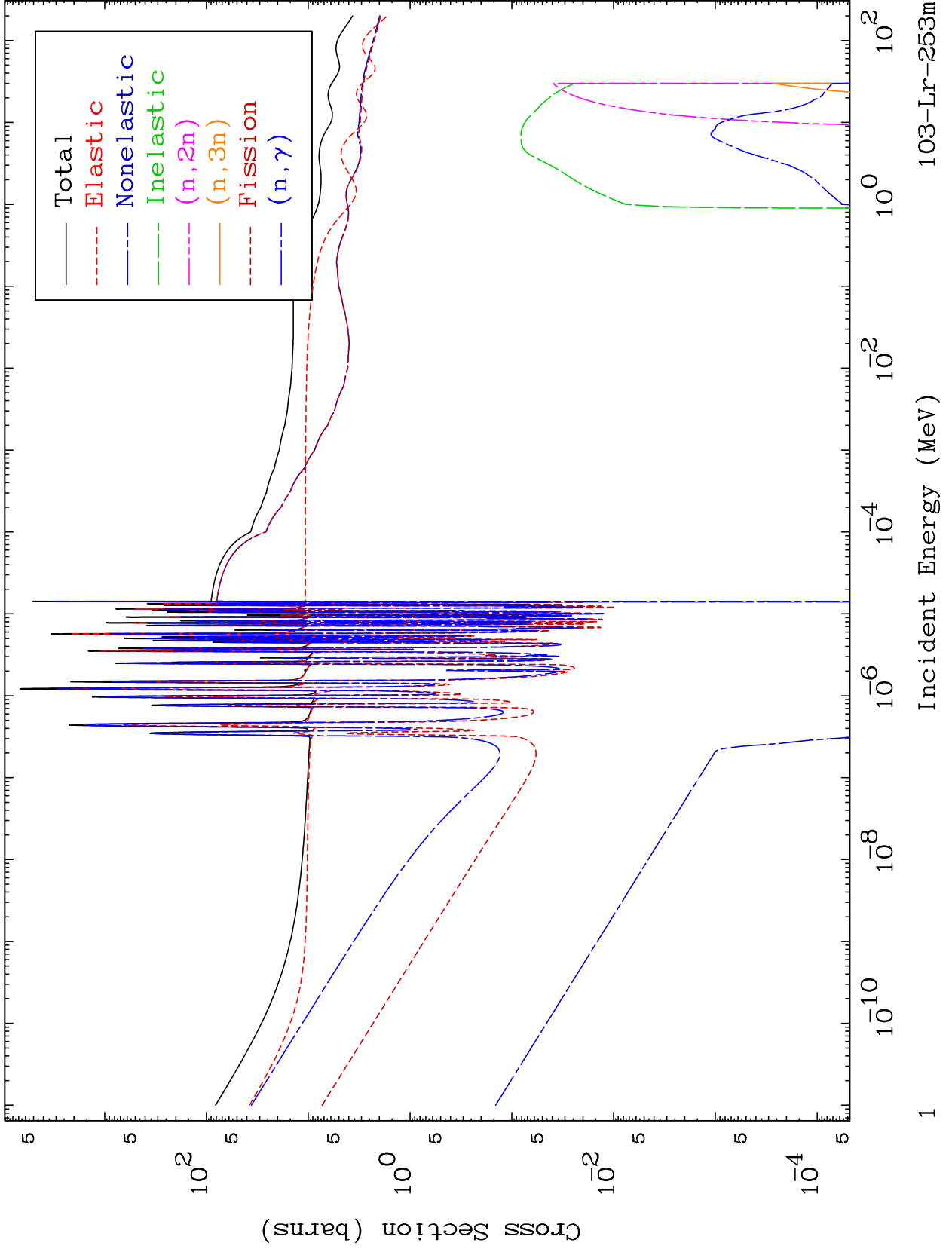
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9987

Neutron Major  
293 Kelvin Cross Sections

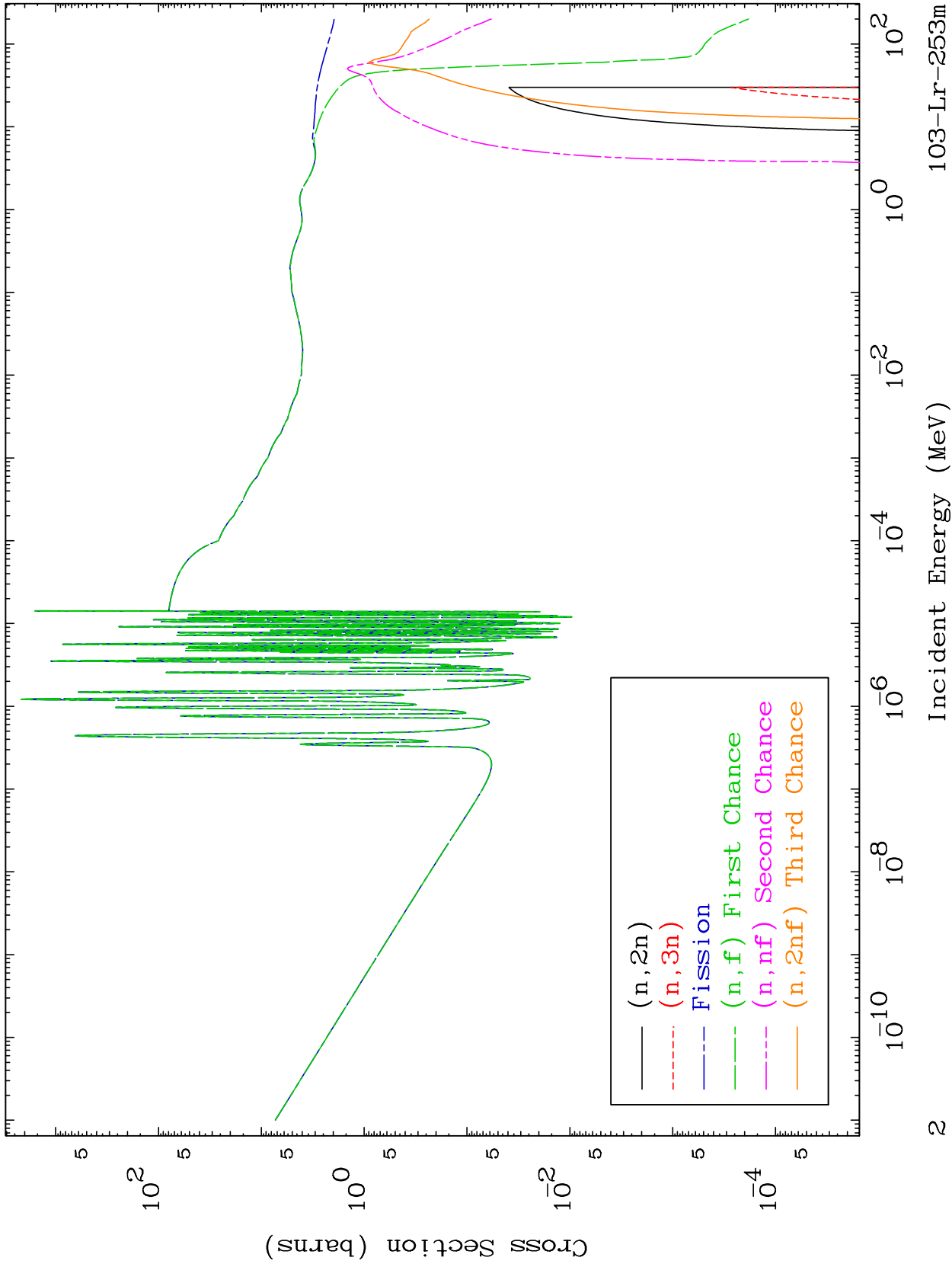
103-Lr-253m



MAT 9987

Neutron Absorption  
293 Kelvin Cross Sections

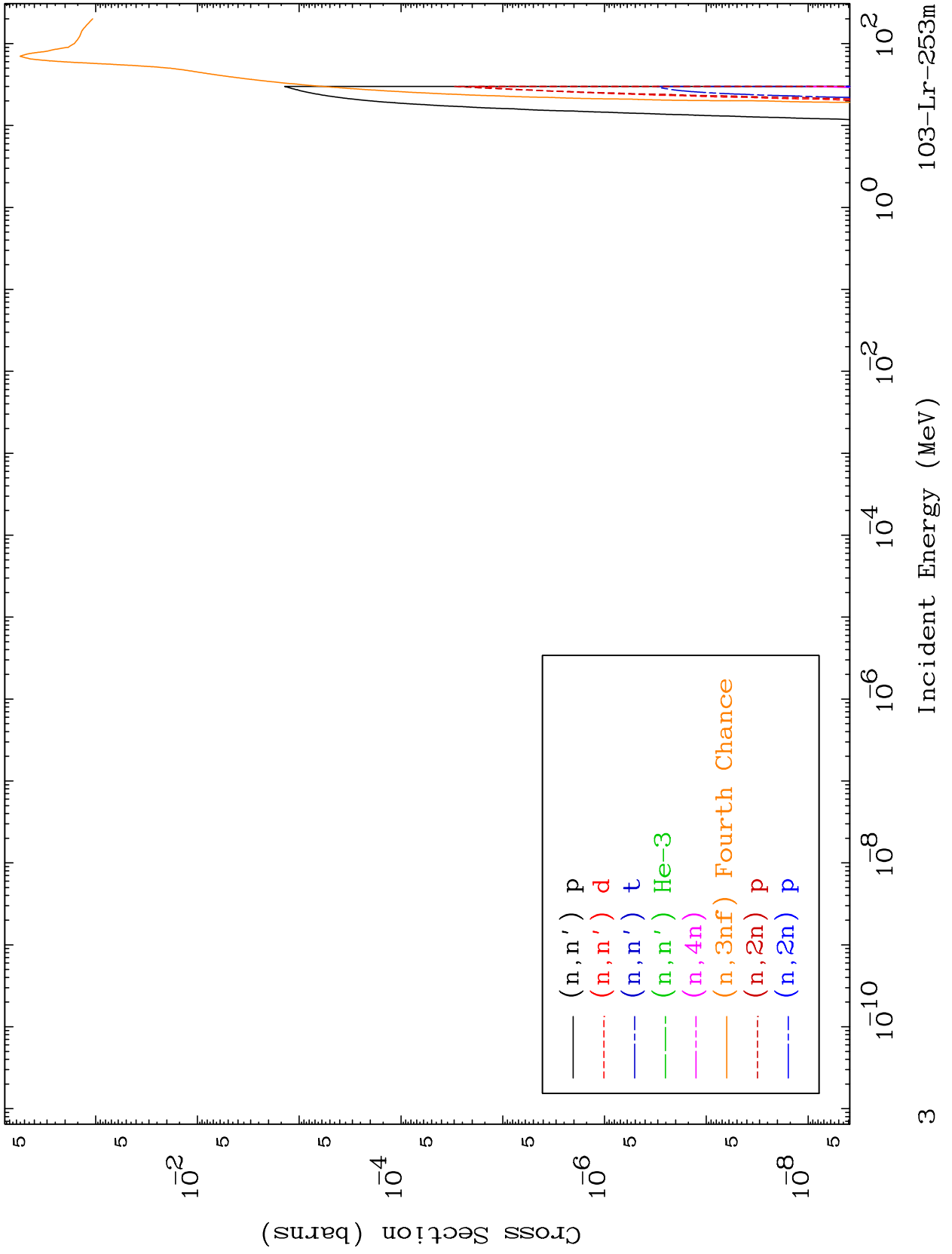
103-Lr-253m



MAT 9987

Neutron Absorption  
293 Kelvin Cross Sections

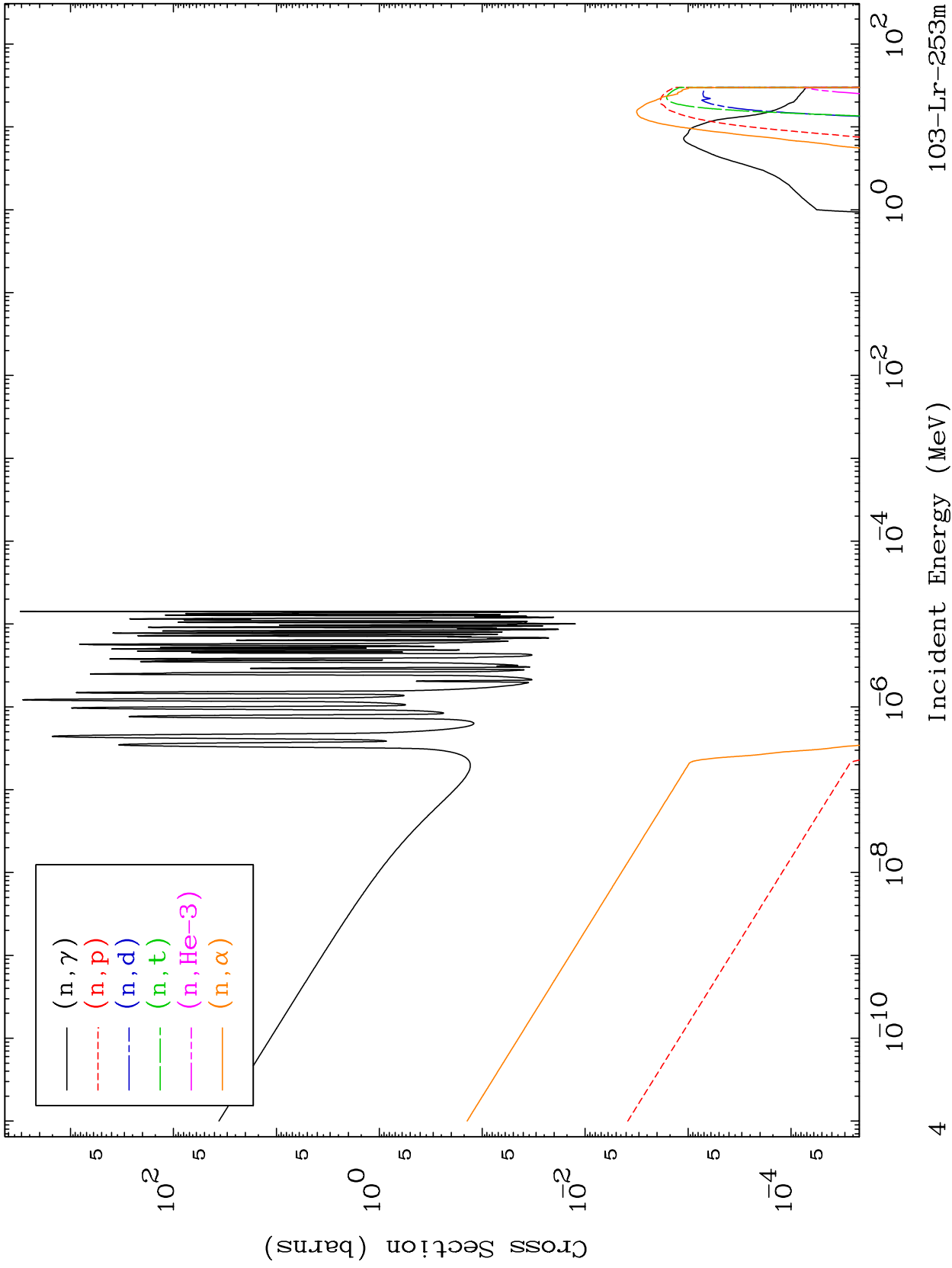
103-Lr-253m

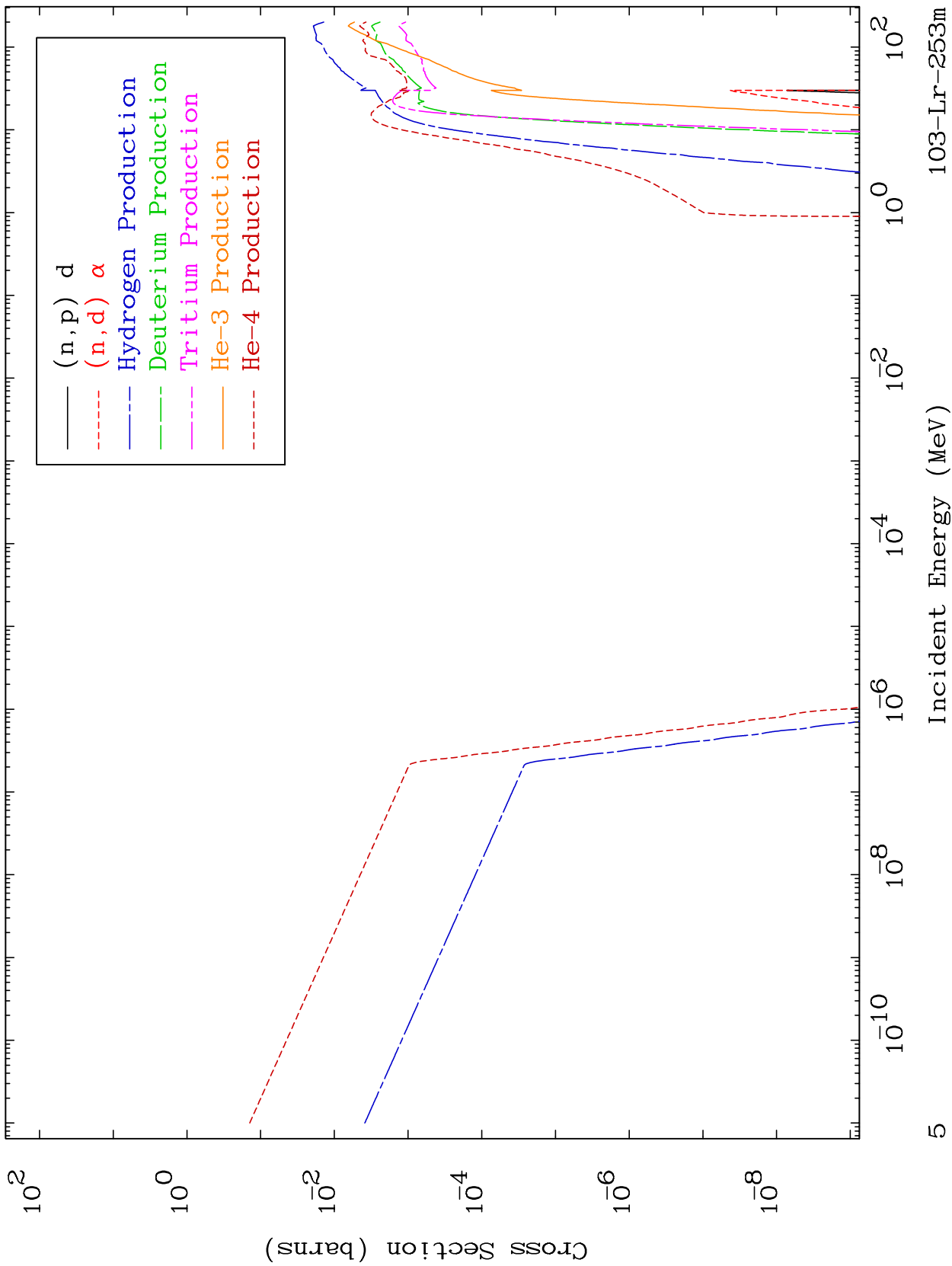


MAT 9987

Neutron Absorption  
293 Kelvin Cross Sections

103-Lr-253m

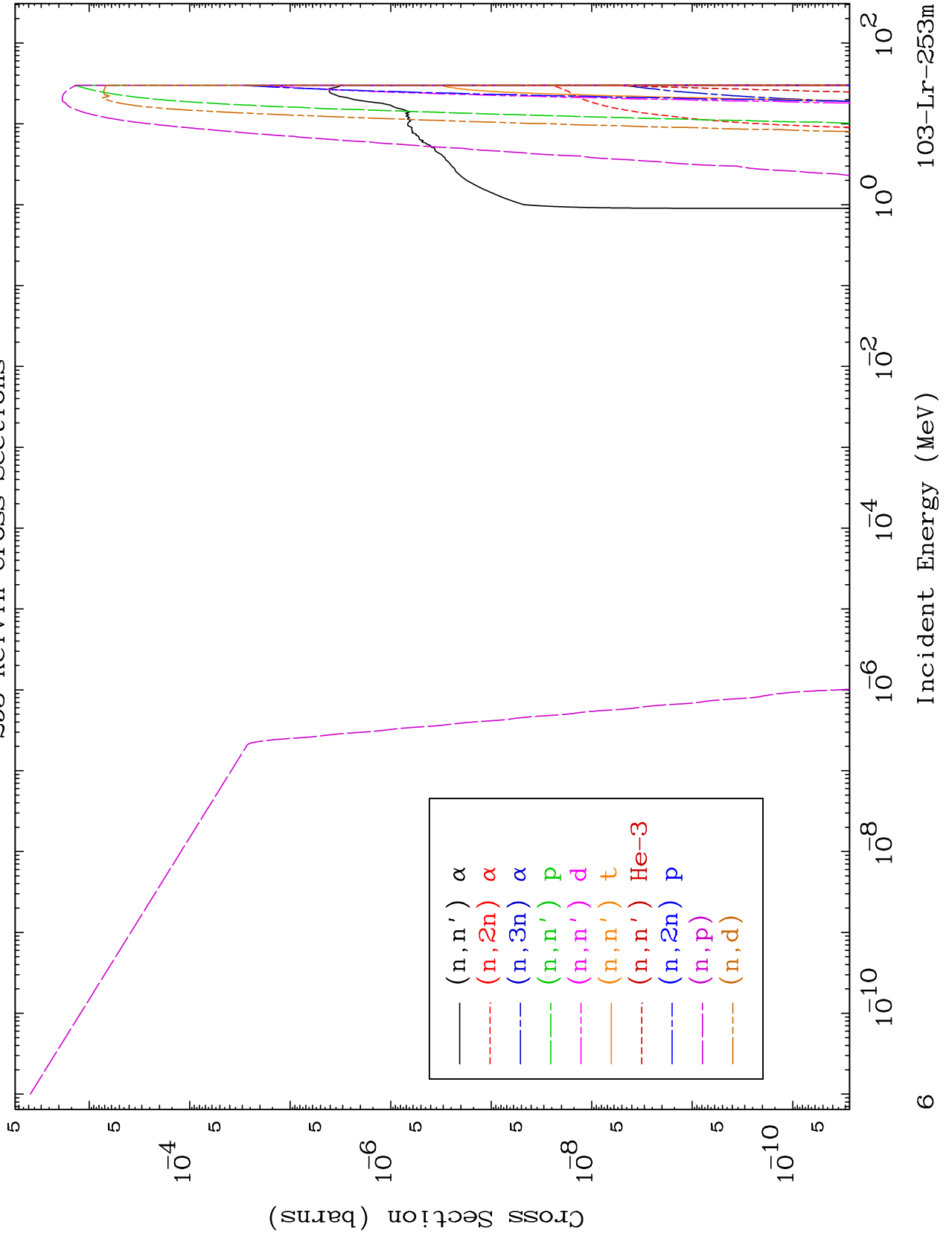




MAT 9987

Charged Particle  
293 Kelvin Cross Sections

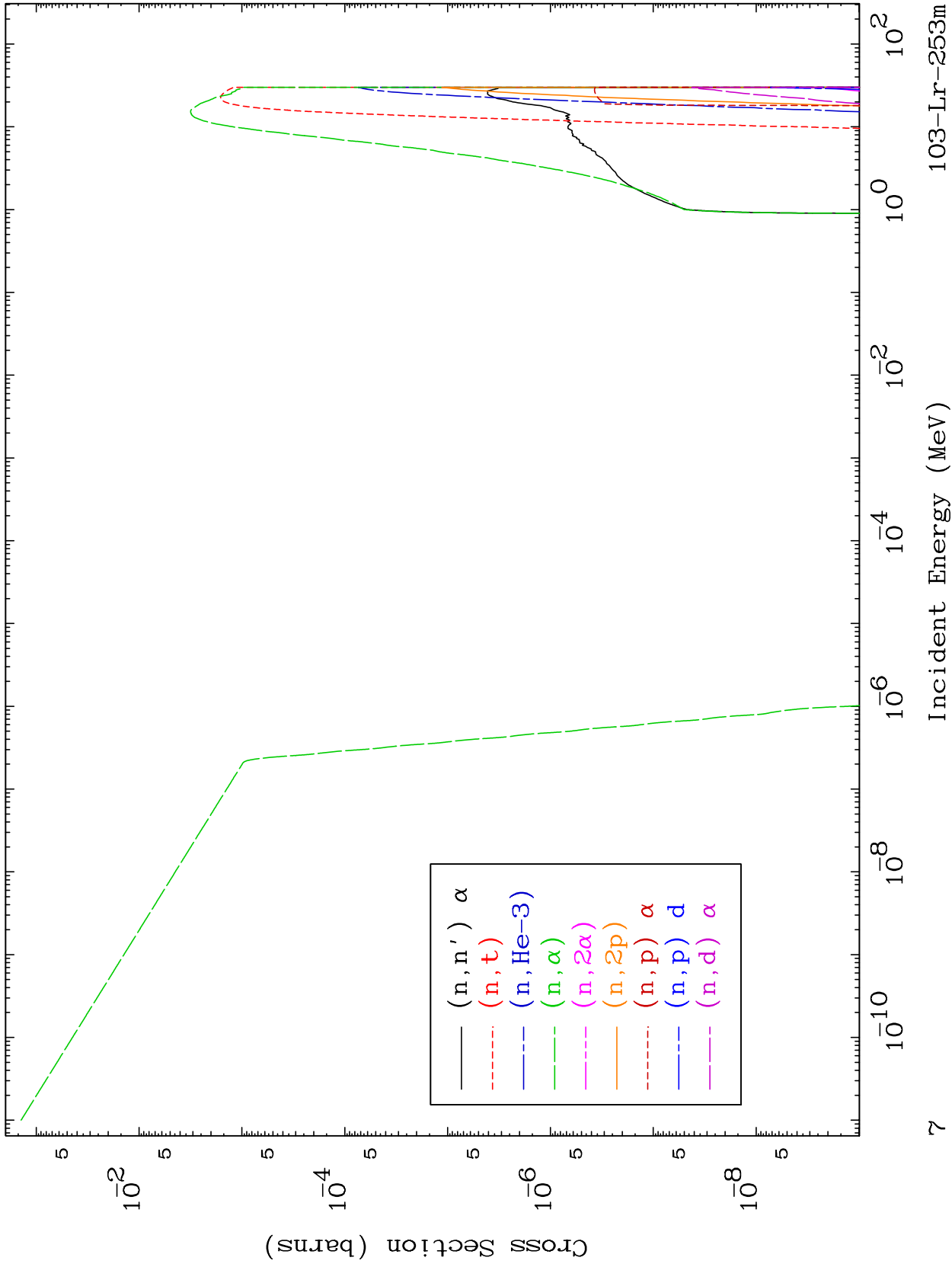
103-Lr-253m



MAT 9987

Charged Particle  
293 Kelvin Cross Sections

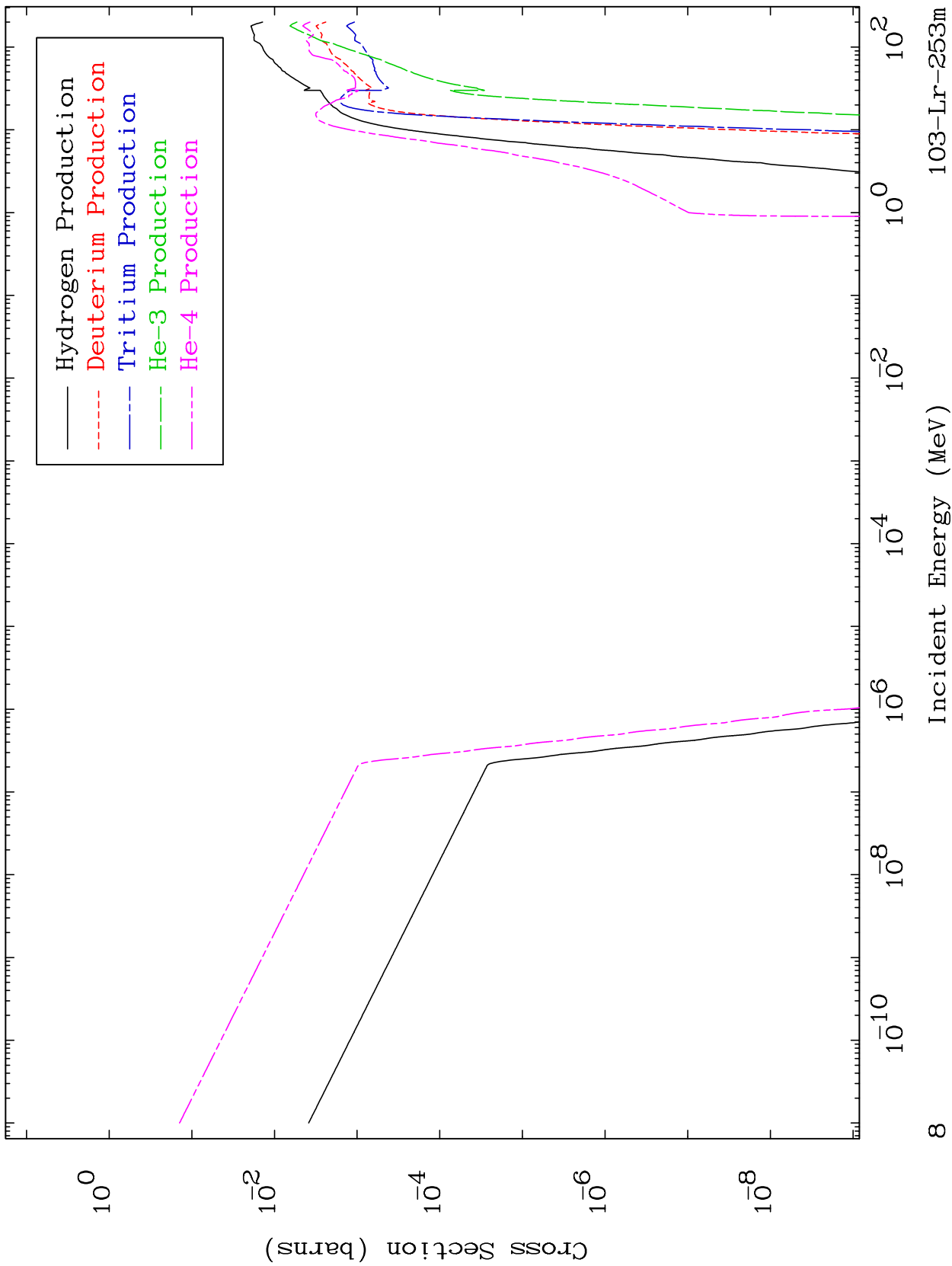
103-Lr-253m



MAT 9987

Particle Production  
293 Kelvin Cross Sections

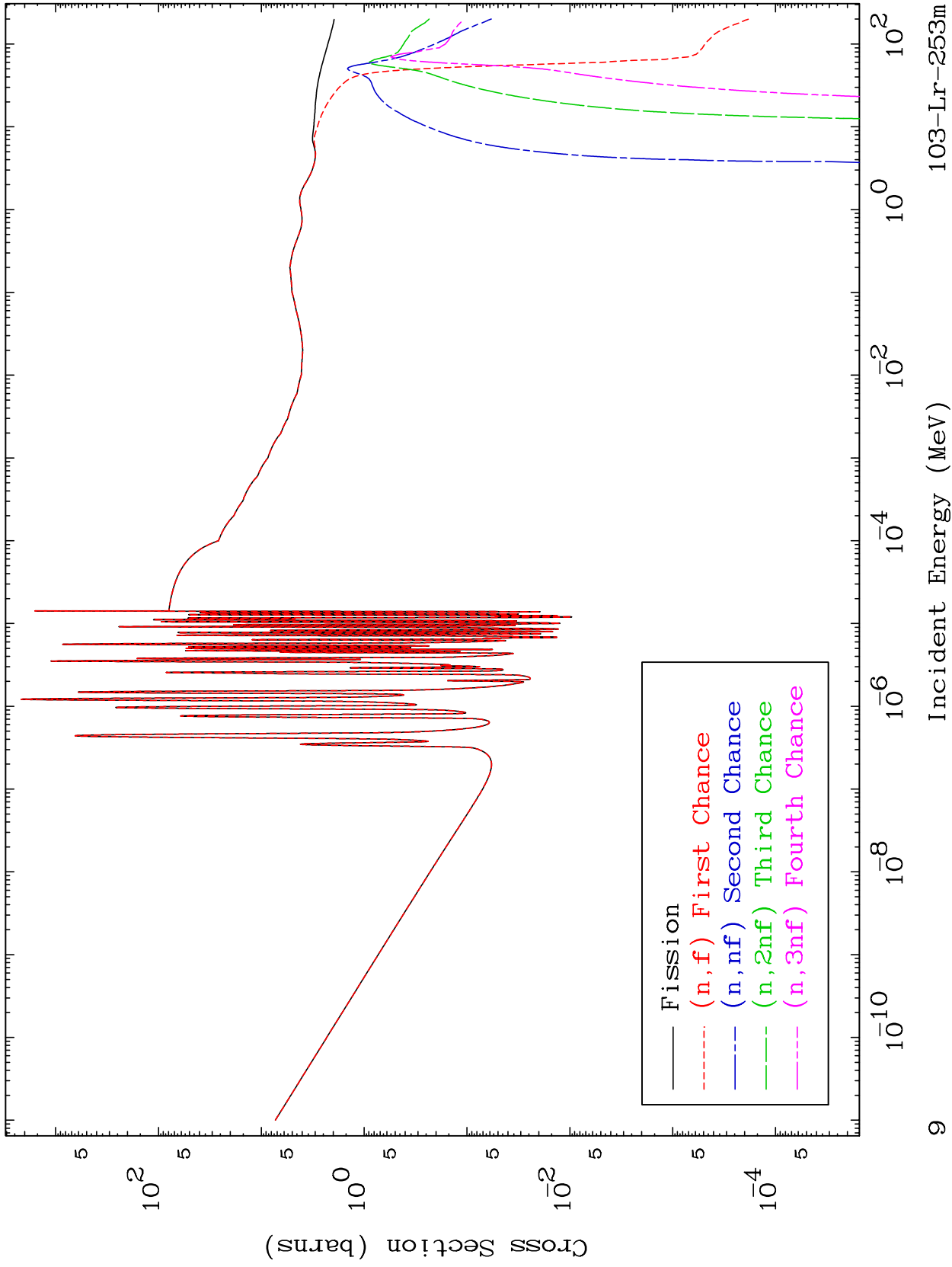
103-Lr-253m



MAT 9987

Fission  
293 Kelvin Cross Sections

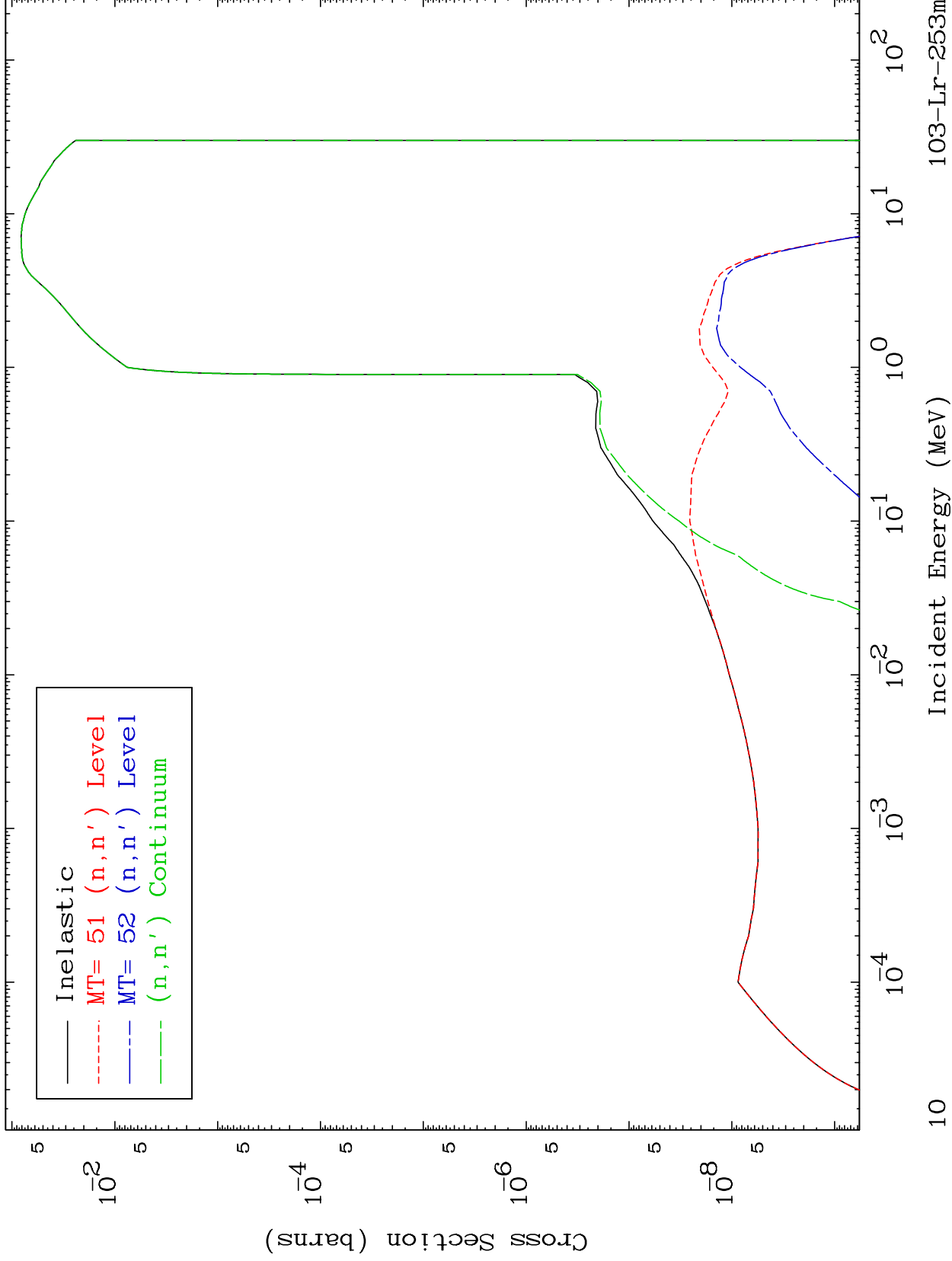
103-Lr-253m



MAT 9987

(n,n') Levels  
293 Kelvin Cross Sections

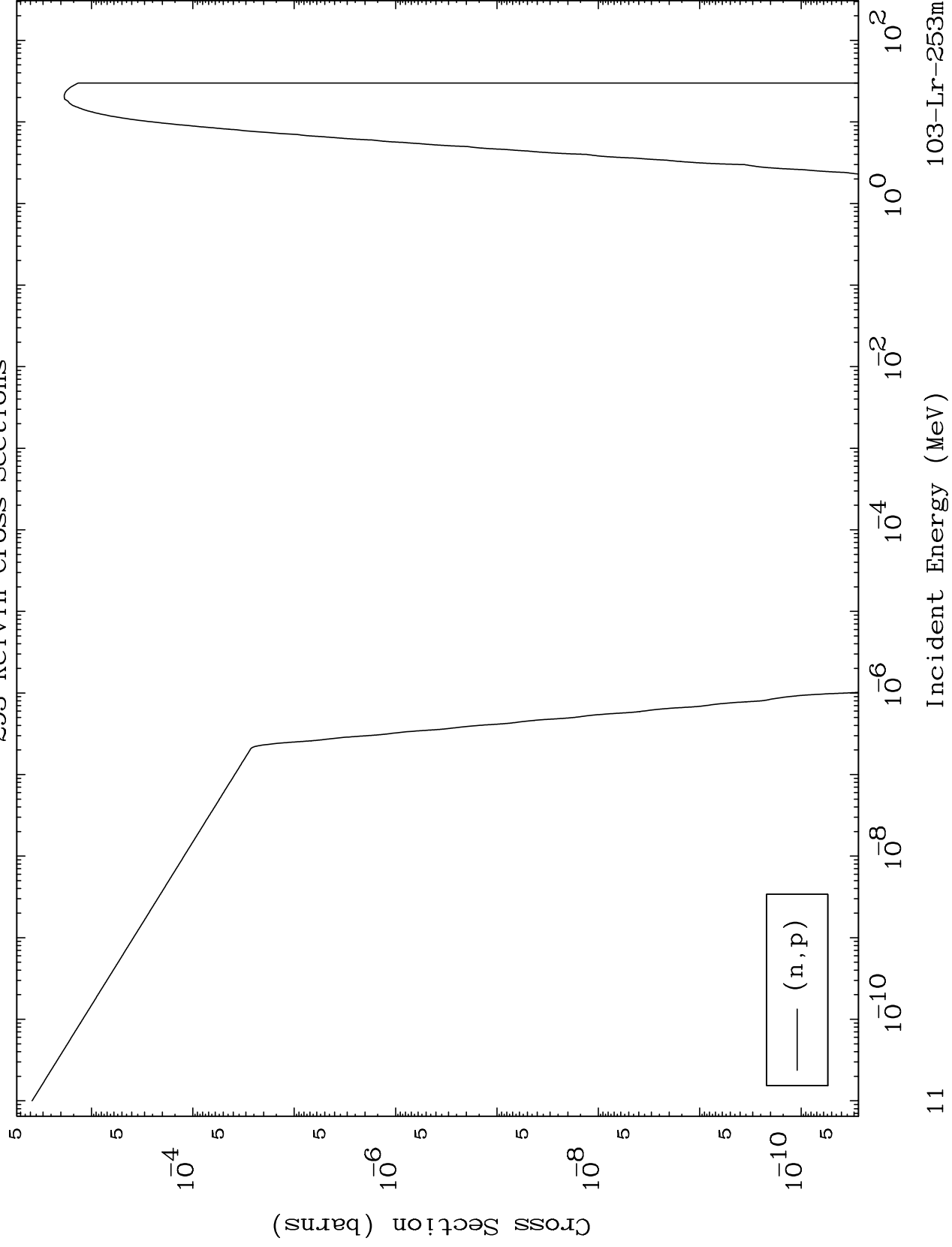
103-Lr-253m



MAT 9987

(n,p) Levels  
293 Kelvin Cross Sections

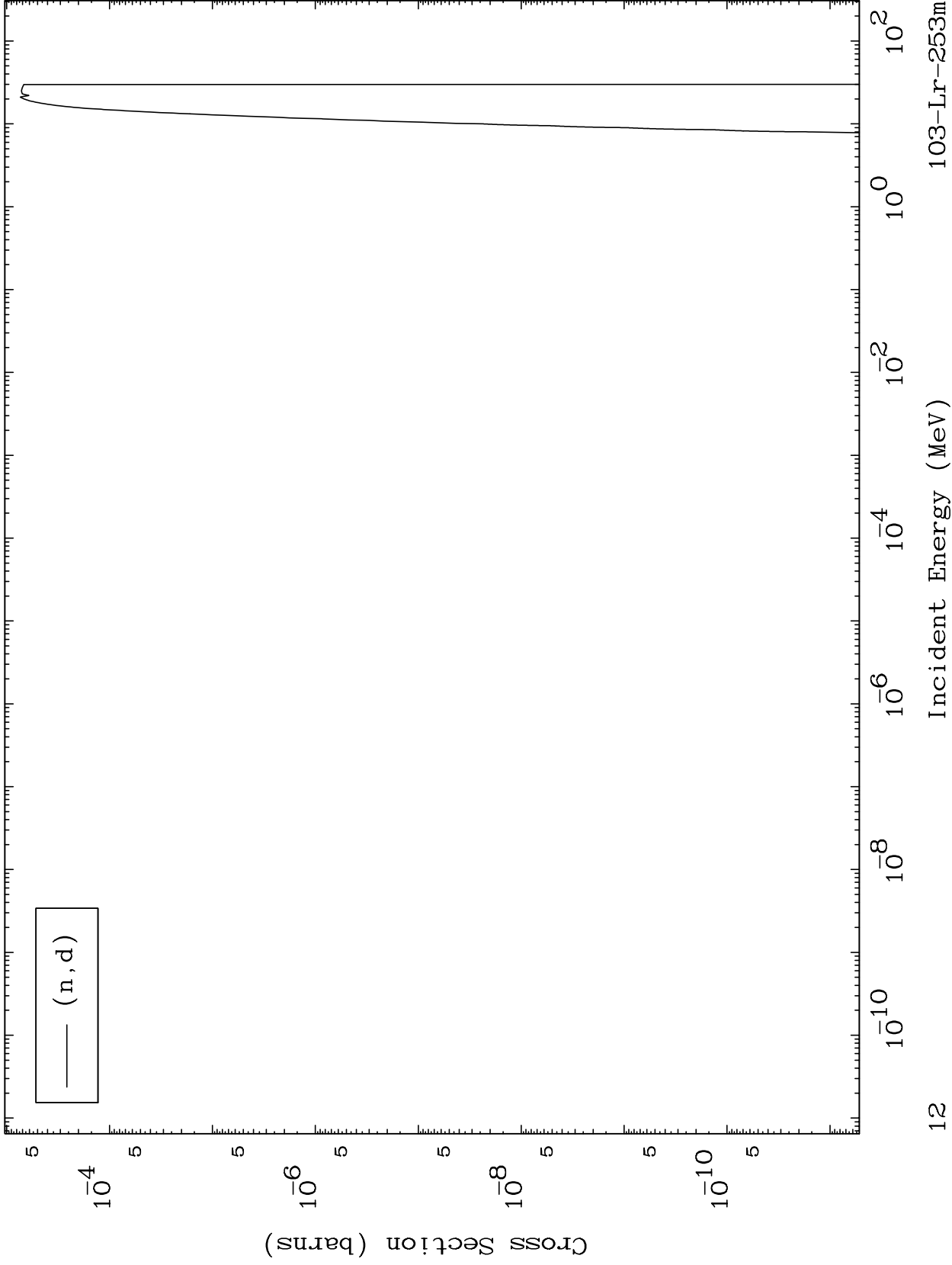
103-Lr-253m



MAT 9987

(n,d) Levels  
293 Kelvin Cross Sections

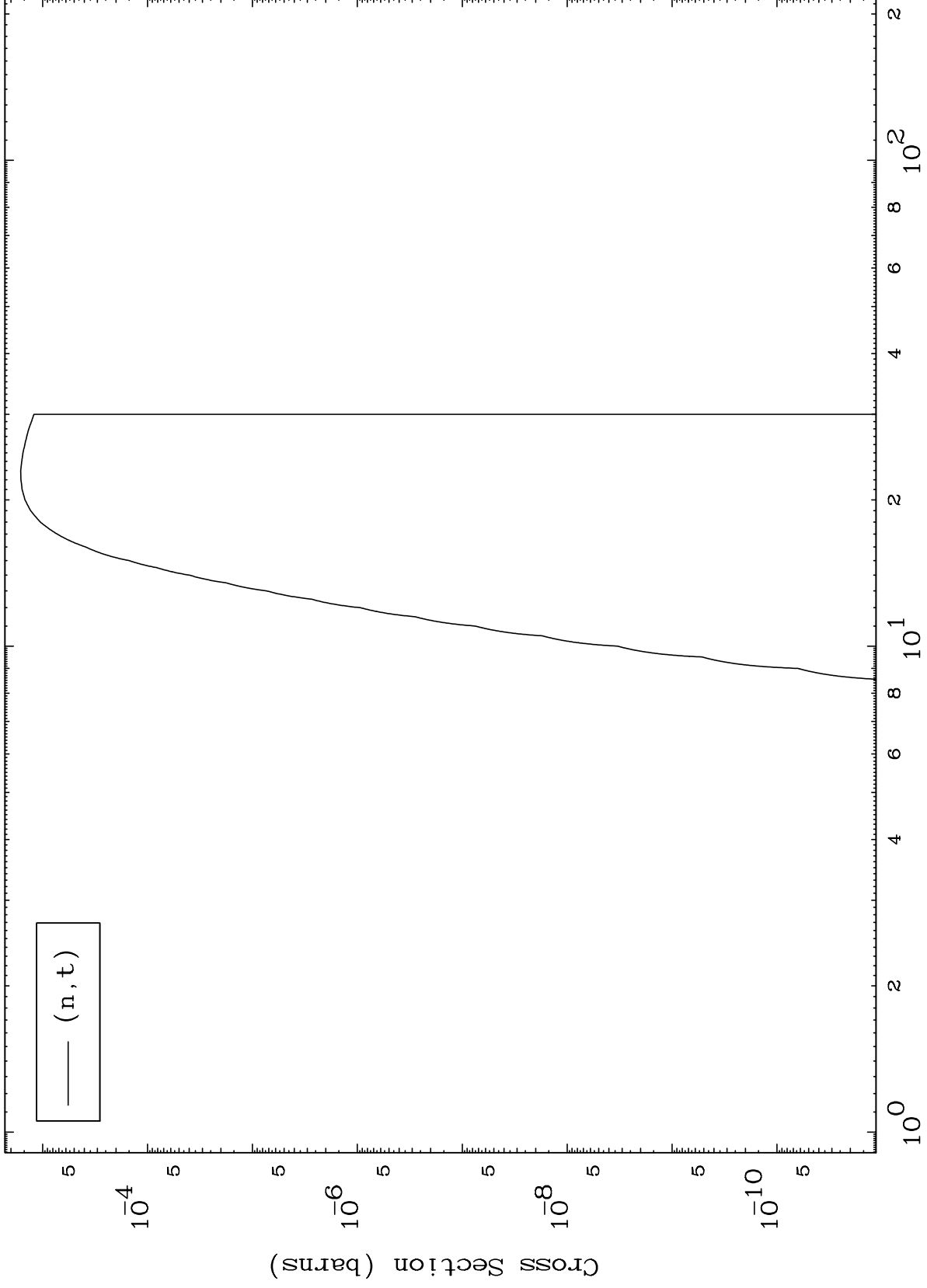
103-Lr-253m



MAT 9987

(n,t) Levels  
293 Kelvin Cross Sections

103-Lr-253m



Incident Energy (MeV)

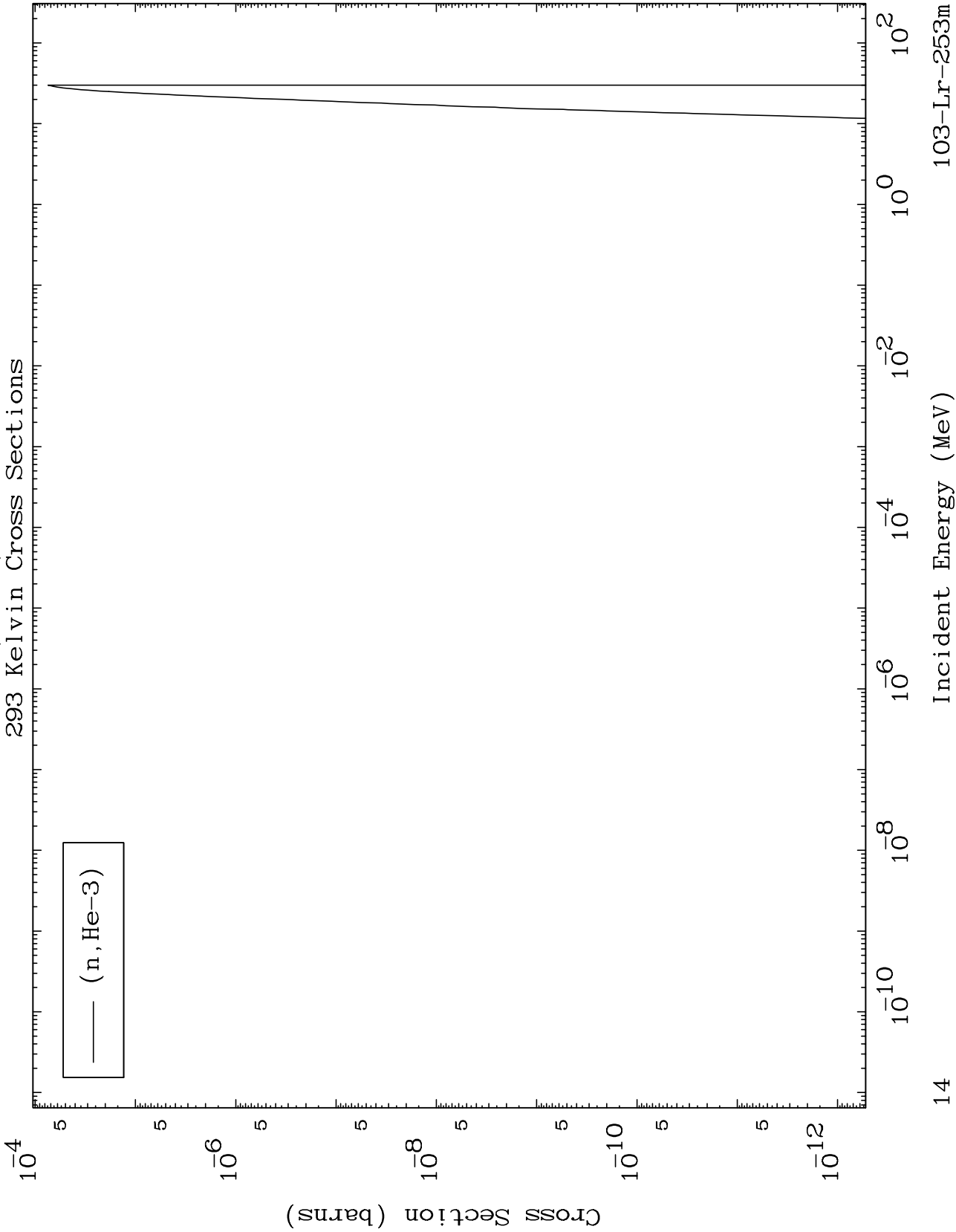
103-Lr-253m

13

MAT 9987

(n,He3) Levels  
293 Kelvin Cross Sections

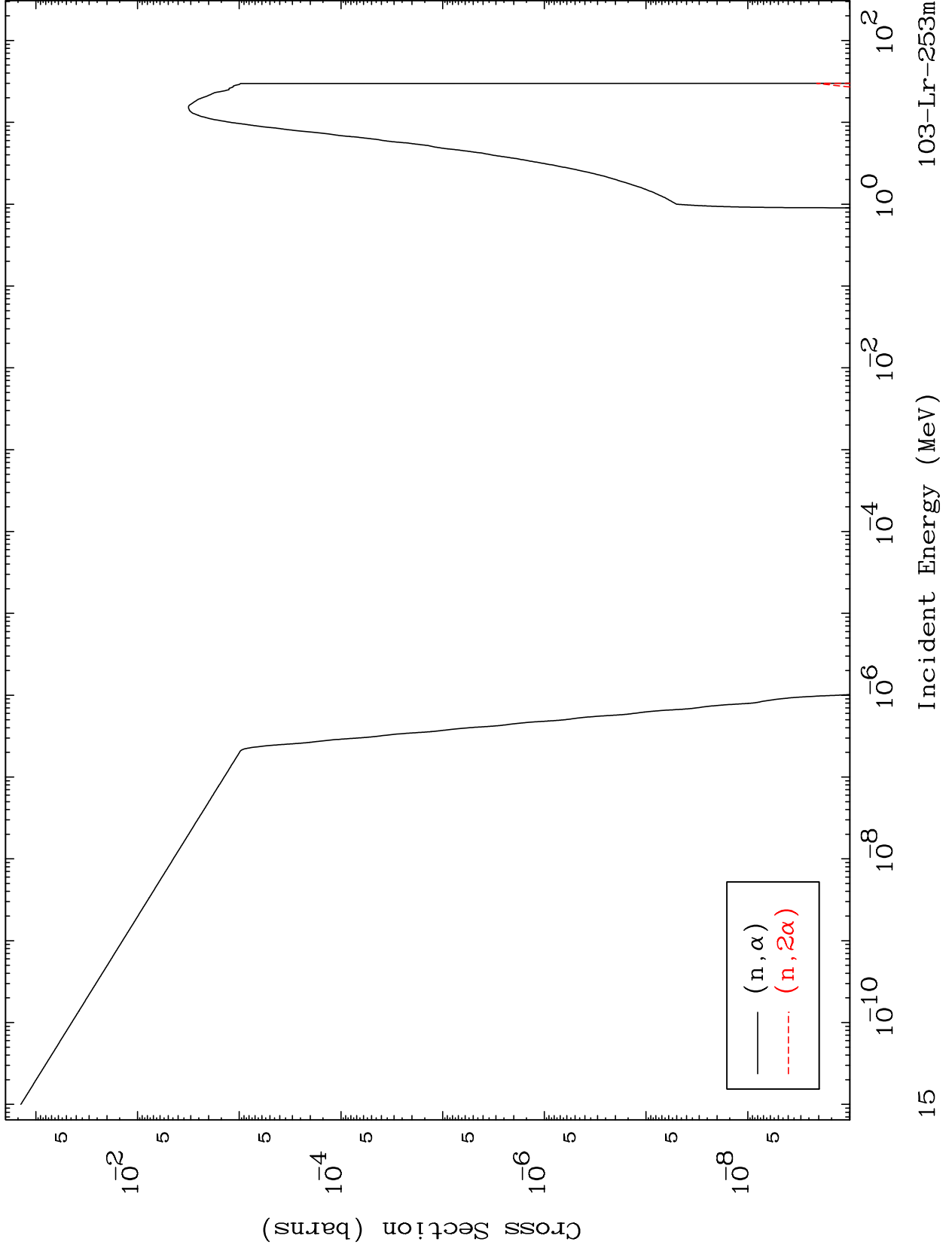
103-Lr-253m



MAT 9987

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

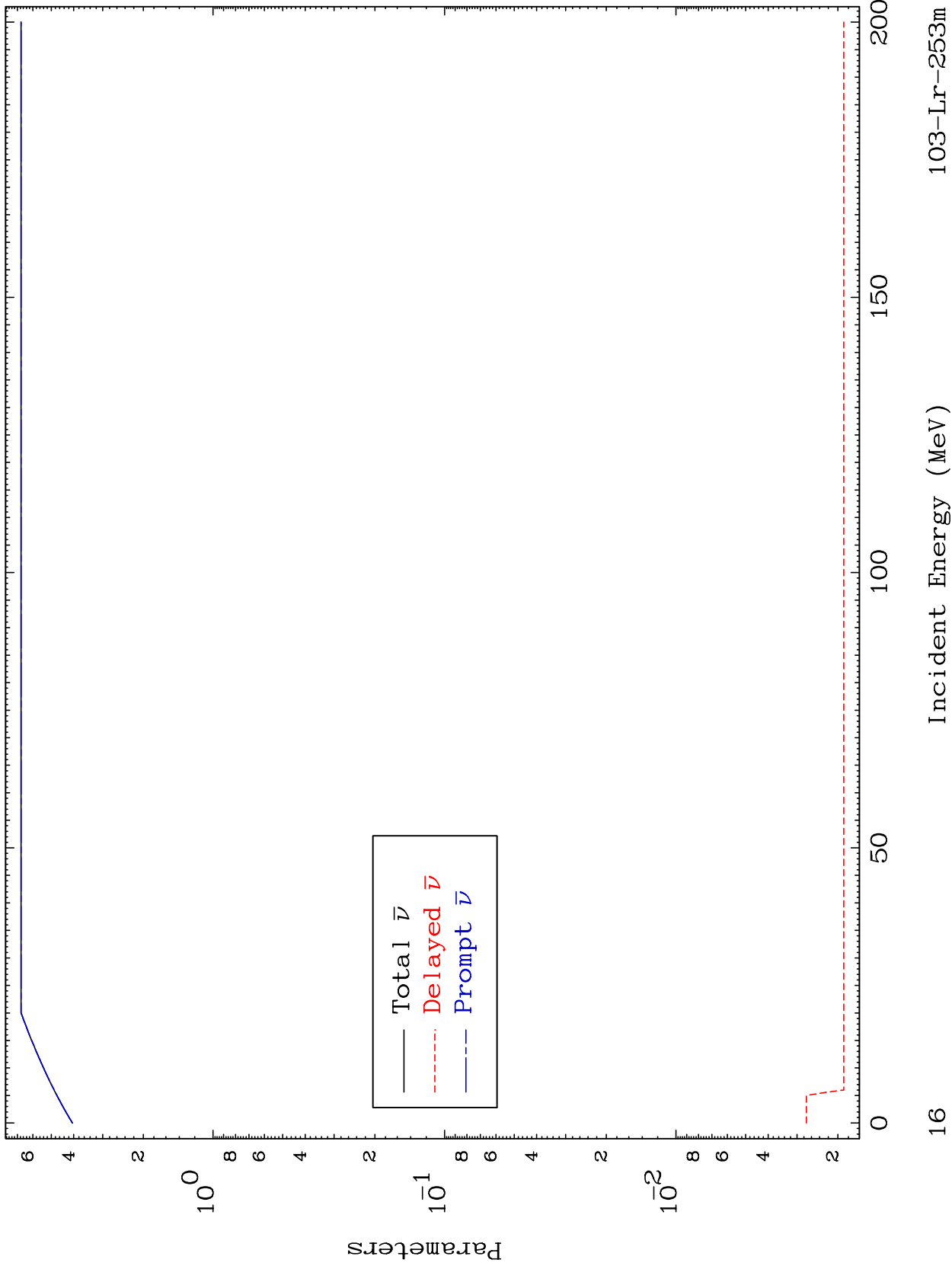
103-Lr-253m



MAT 9987

Energy Release  
Parameters

103-Lr-253m



16

Incident Energy (MeV)

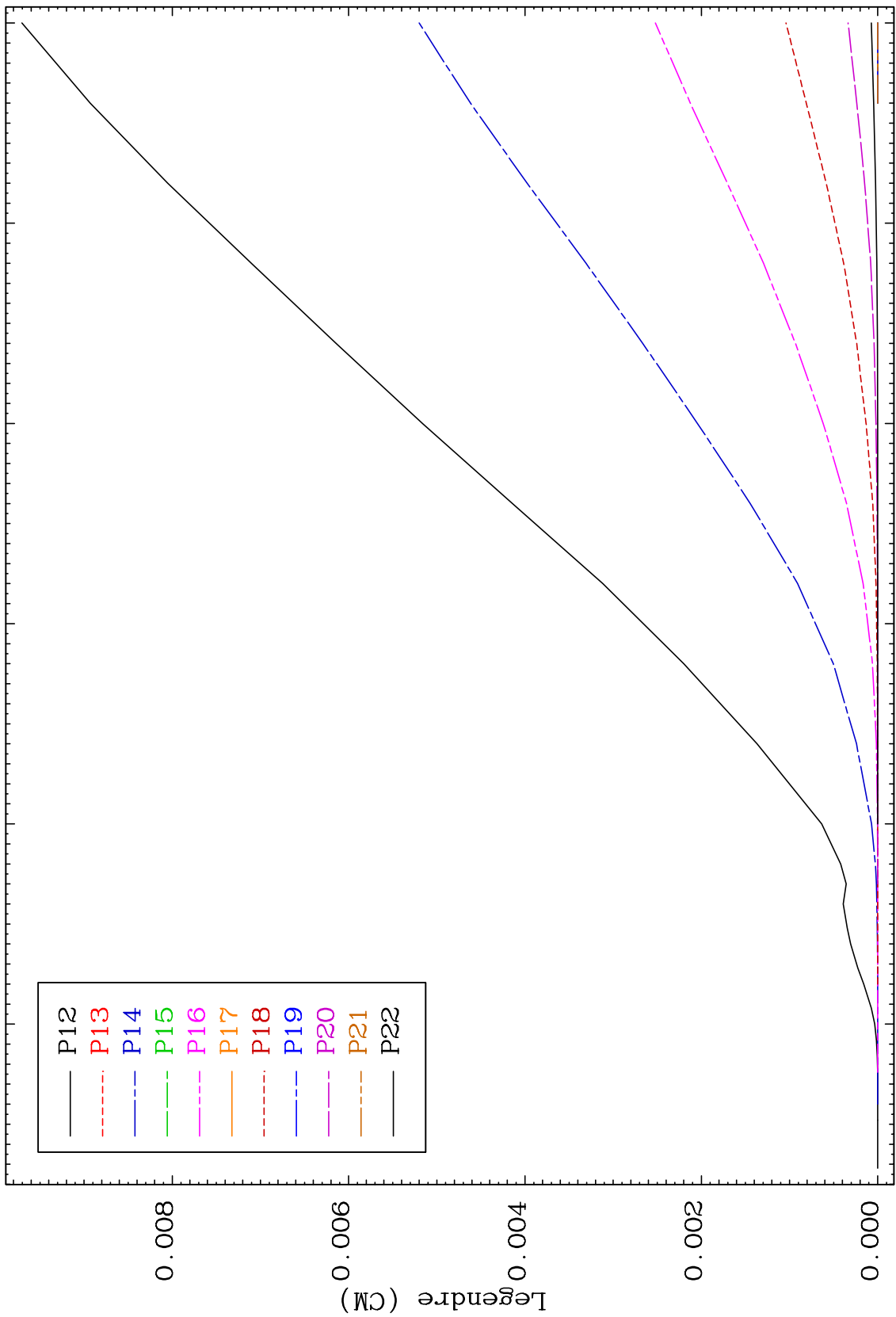
103-Lr-253m



MAT 9987

Elastic Legendre Coefficients

103-Lr-253m



18

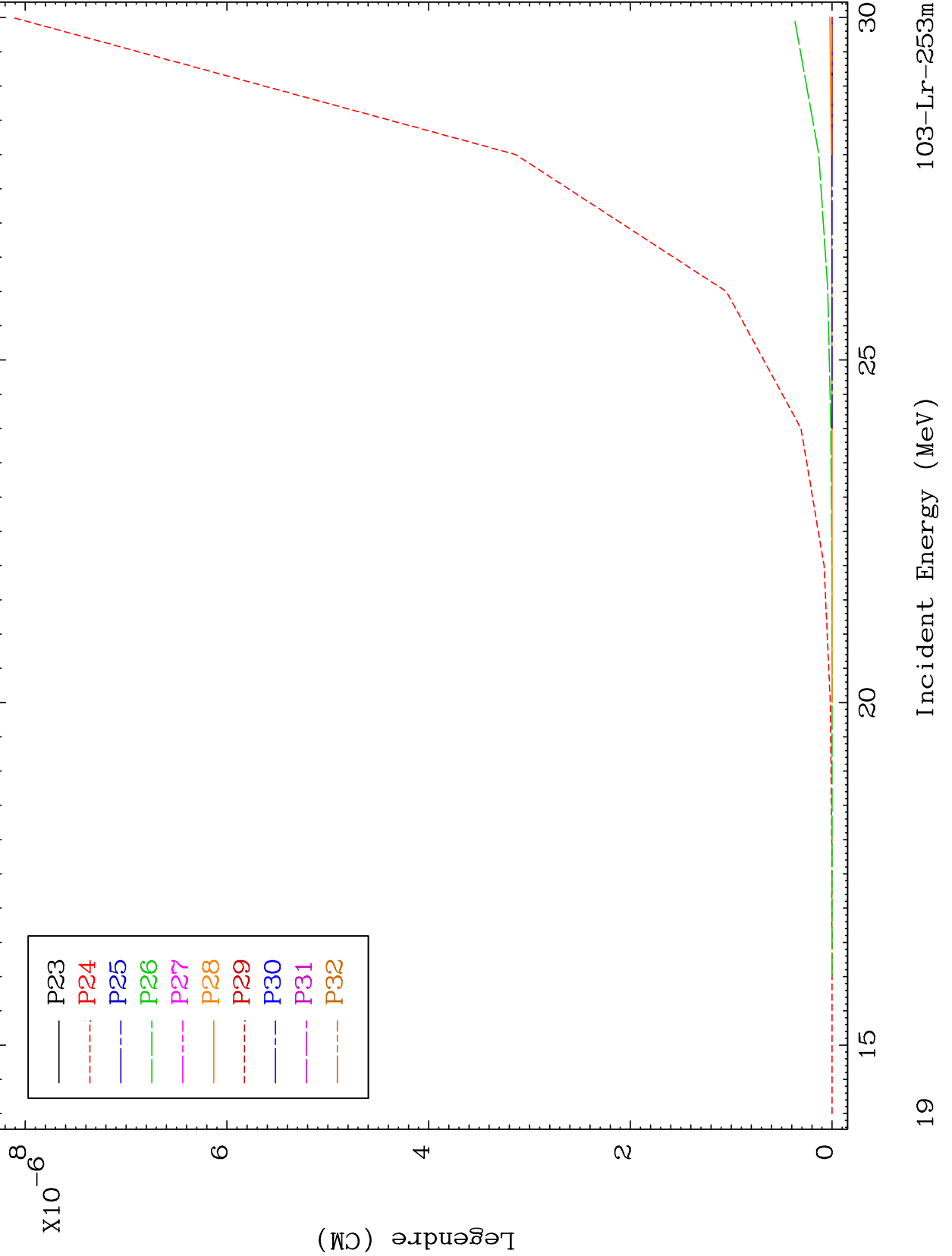
103-Lr-253m

Incident Energy (MeV)

MAT 9987

Elastic Legendre Coefficients

103-Lr-253m



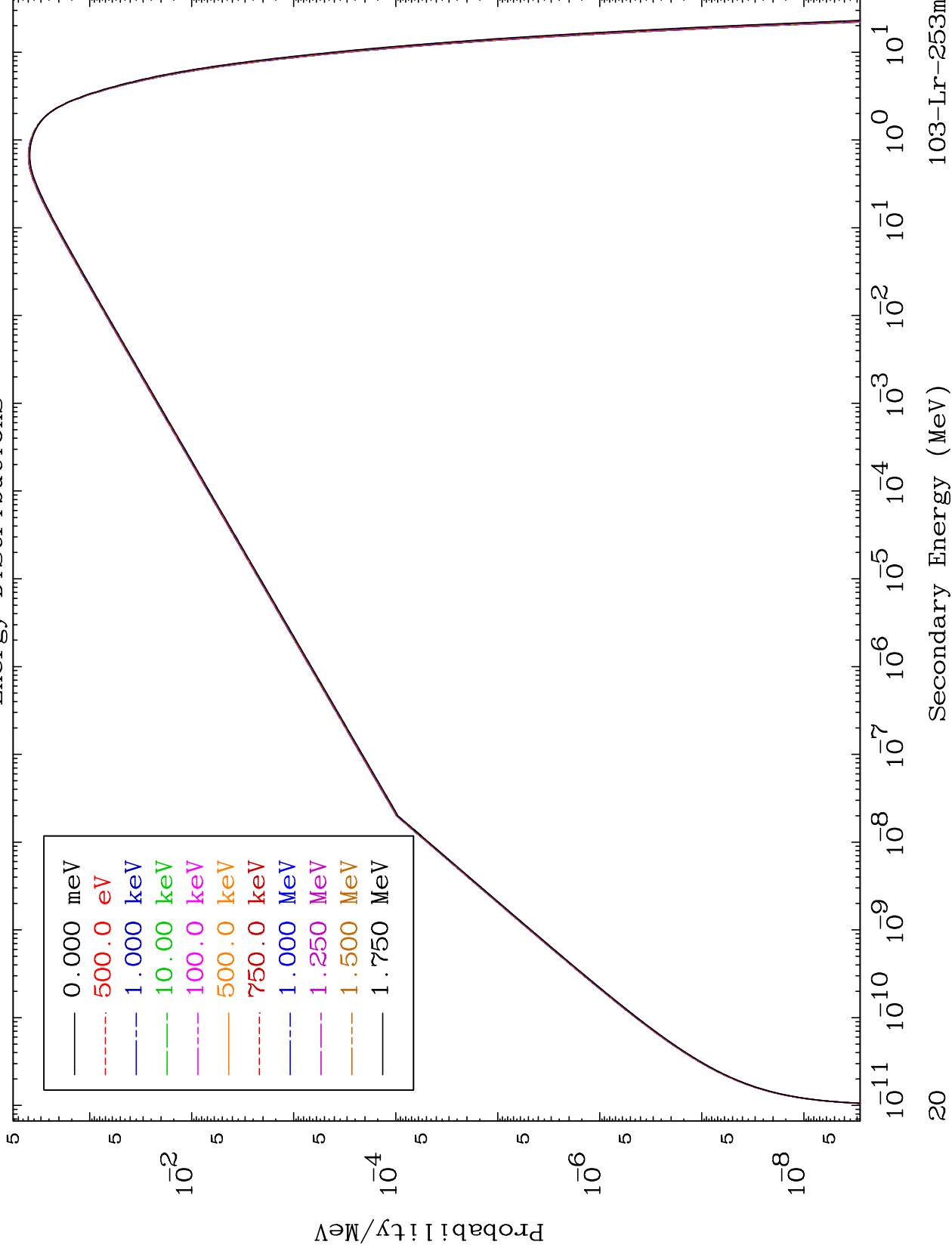
19

103-Lr-253m

MAT 9987

Fission Energy Distributions

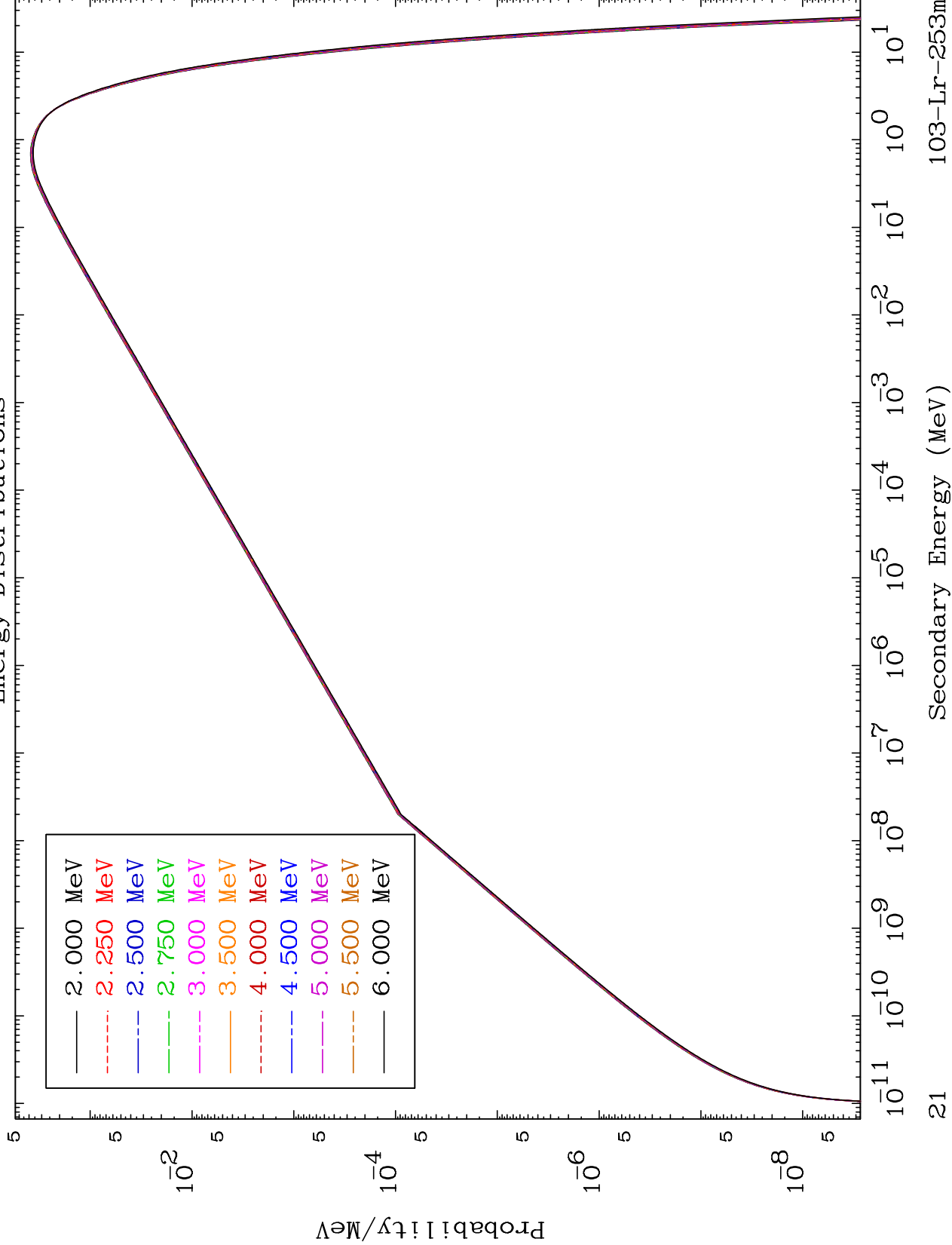
103-Lr-253m



MAT 9987

Fission Energy Distributions

<sup>103</sup>Lr-253m

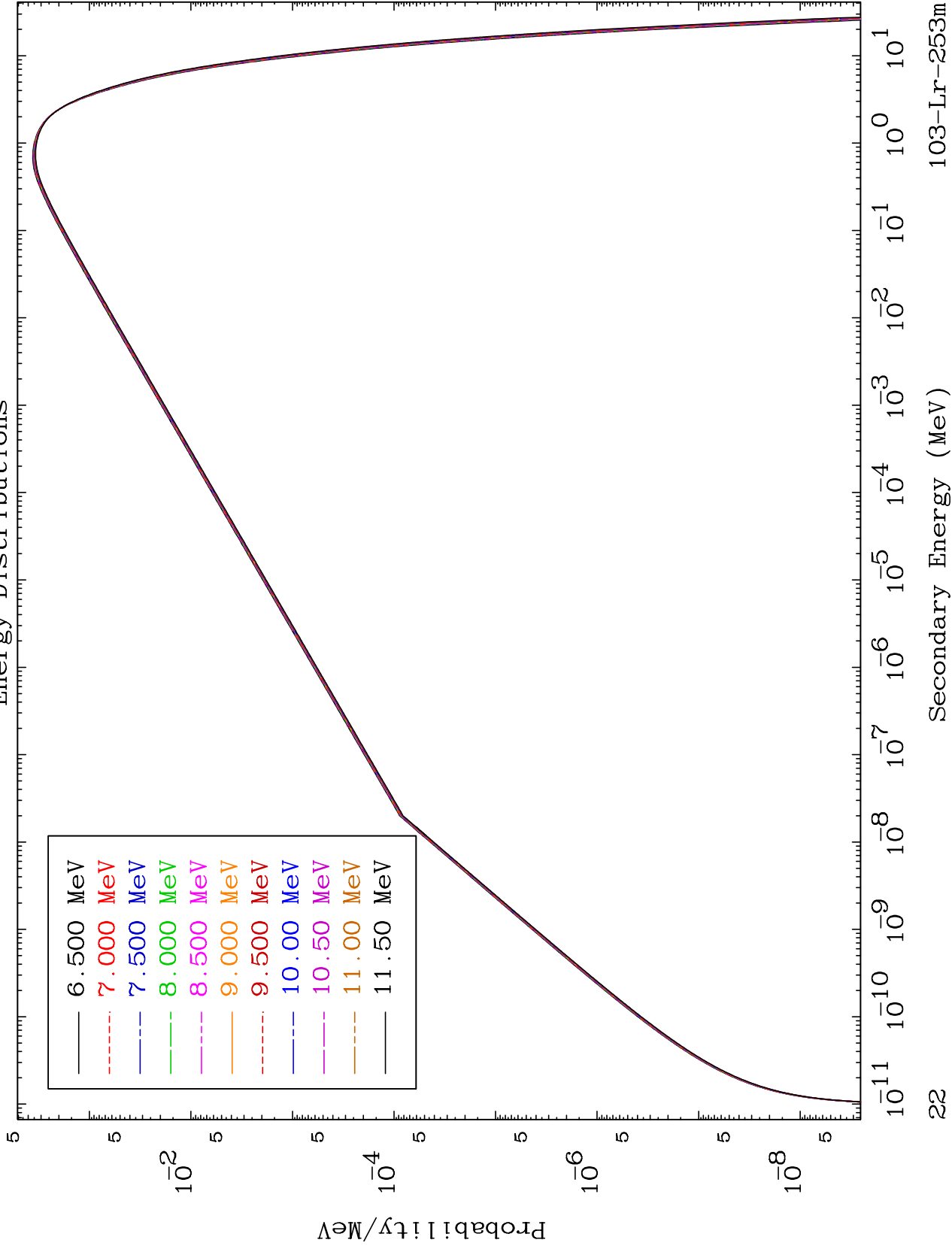


<sup>103</sup>Lr-253m

MAT 9987

Fission  
Energy Distributions

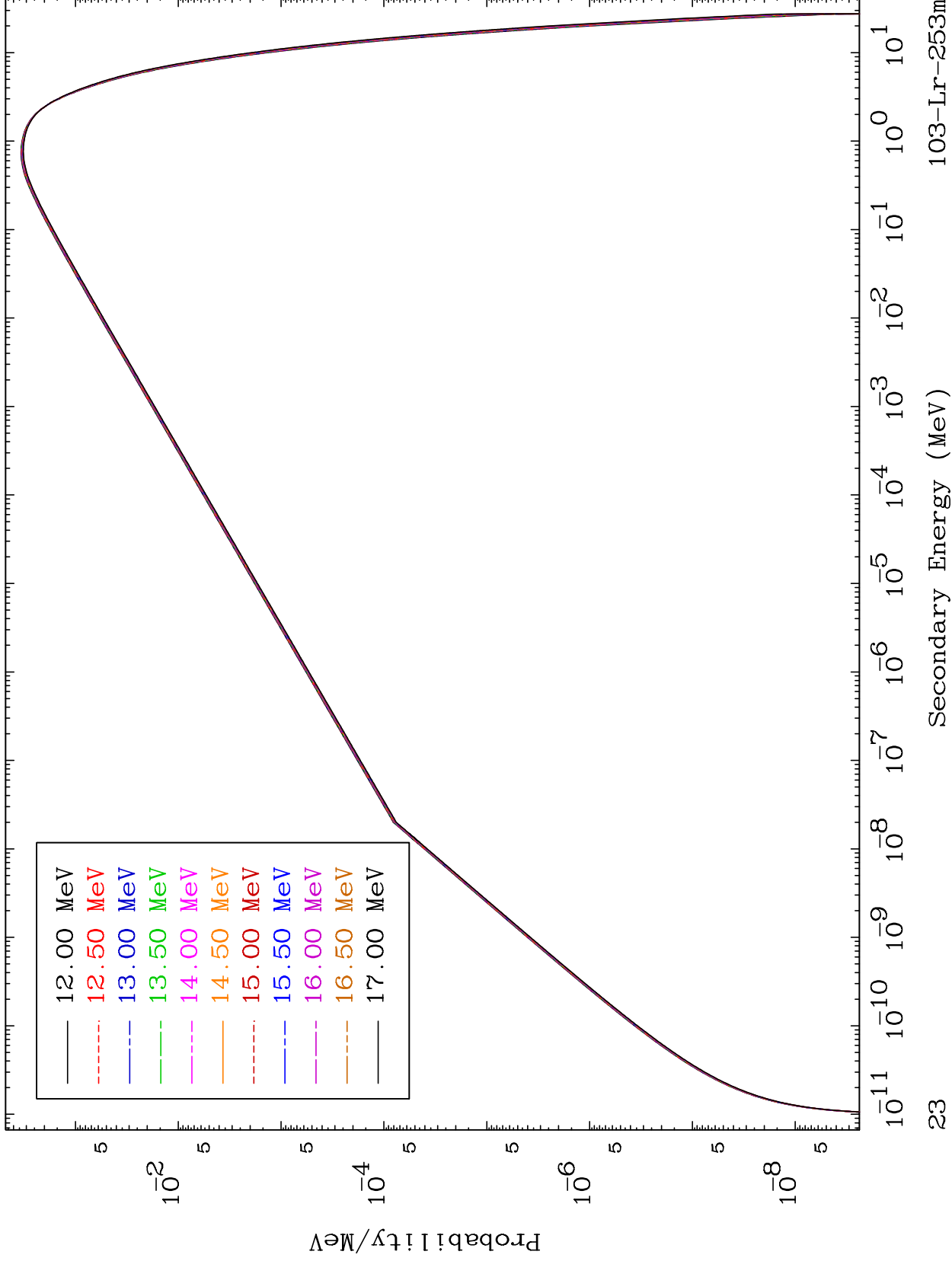
103-Lr-253m



MAT 9987

# Fission Energy Distributions

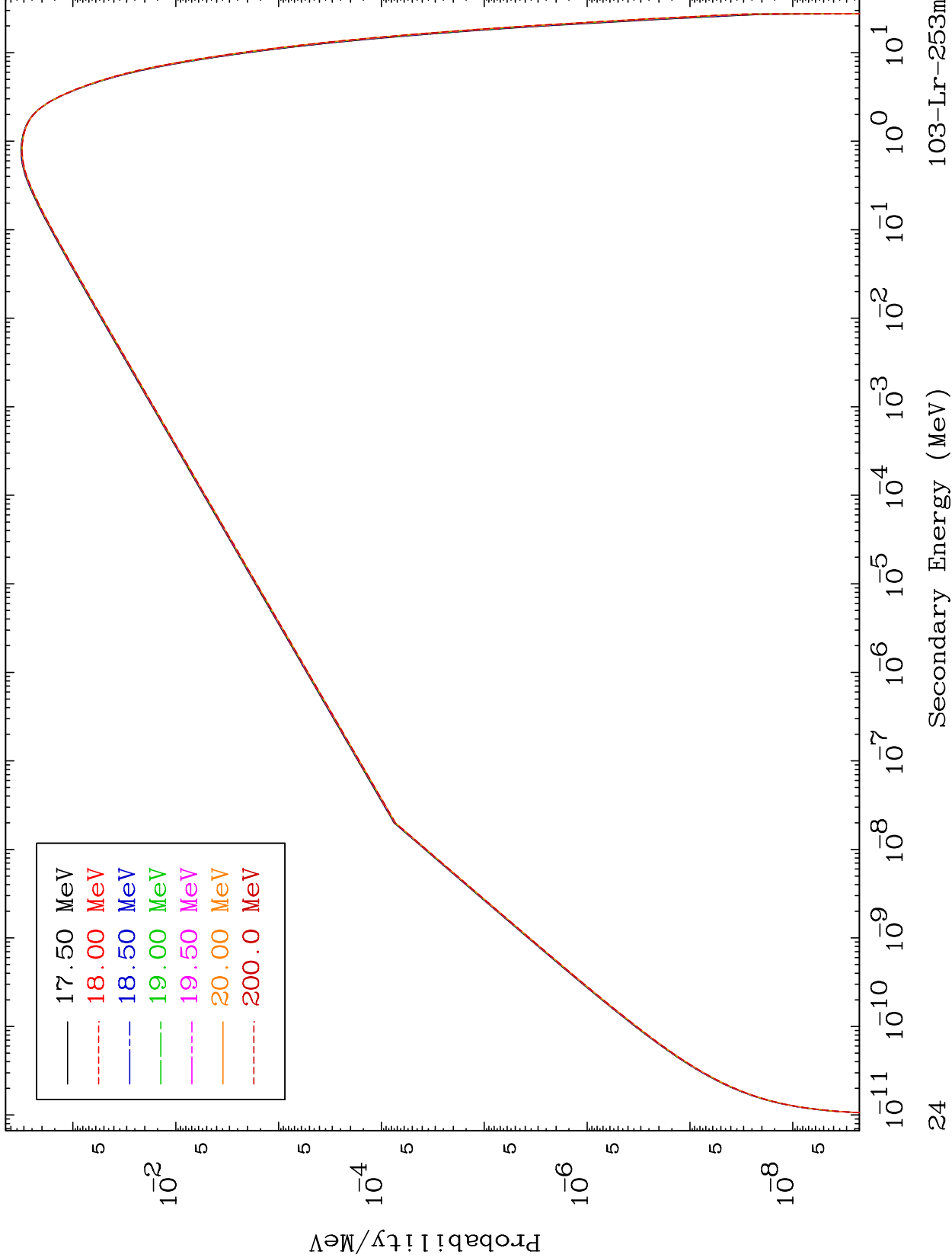
<sup>103</sup>Lr-253m



MAT 9987

Fission Energy Distributions

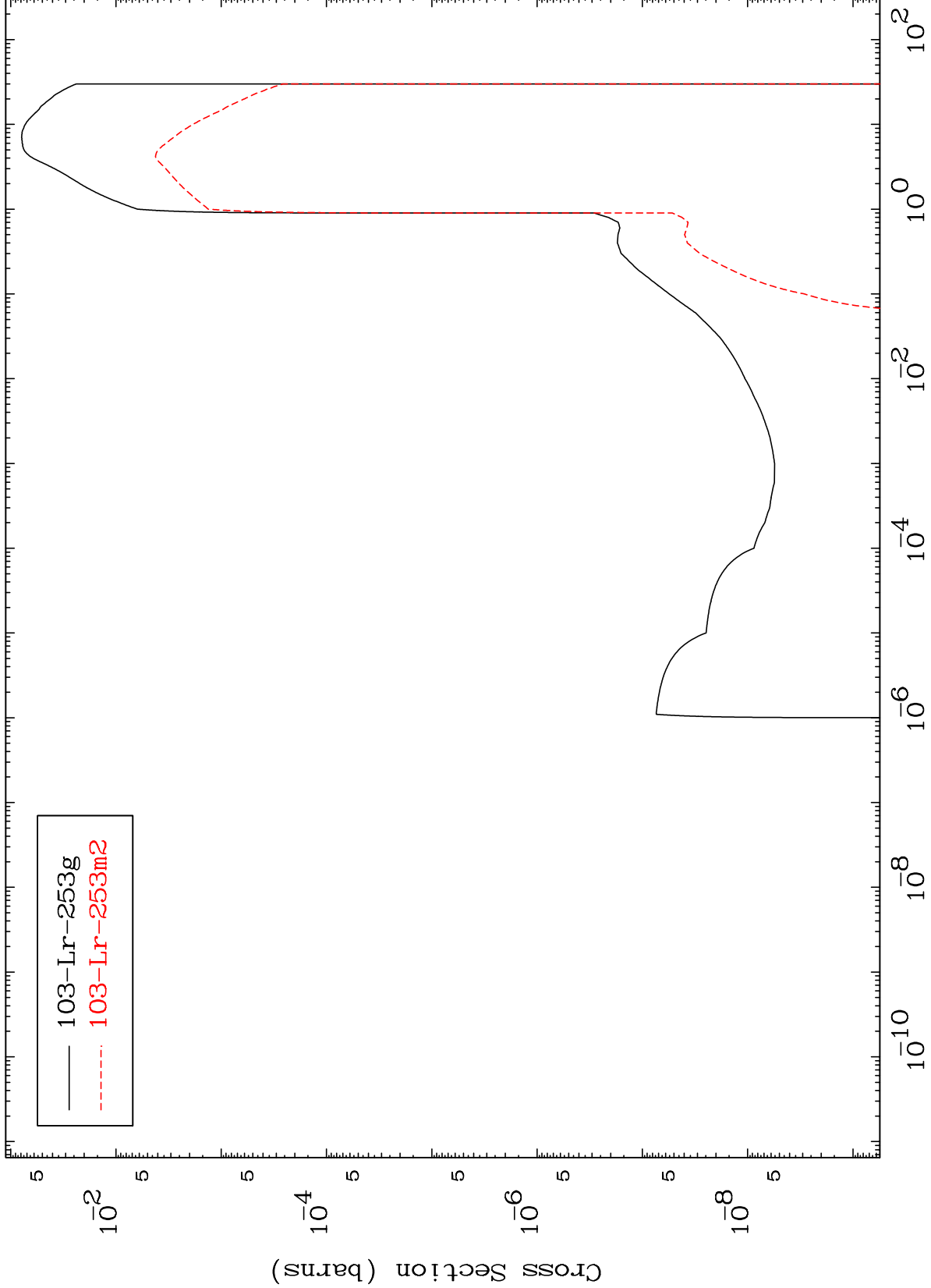
103-Lr-253m



MAT 9987

Inelastic  
Radionuclide Production Cross Section

103-Lr-253m



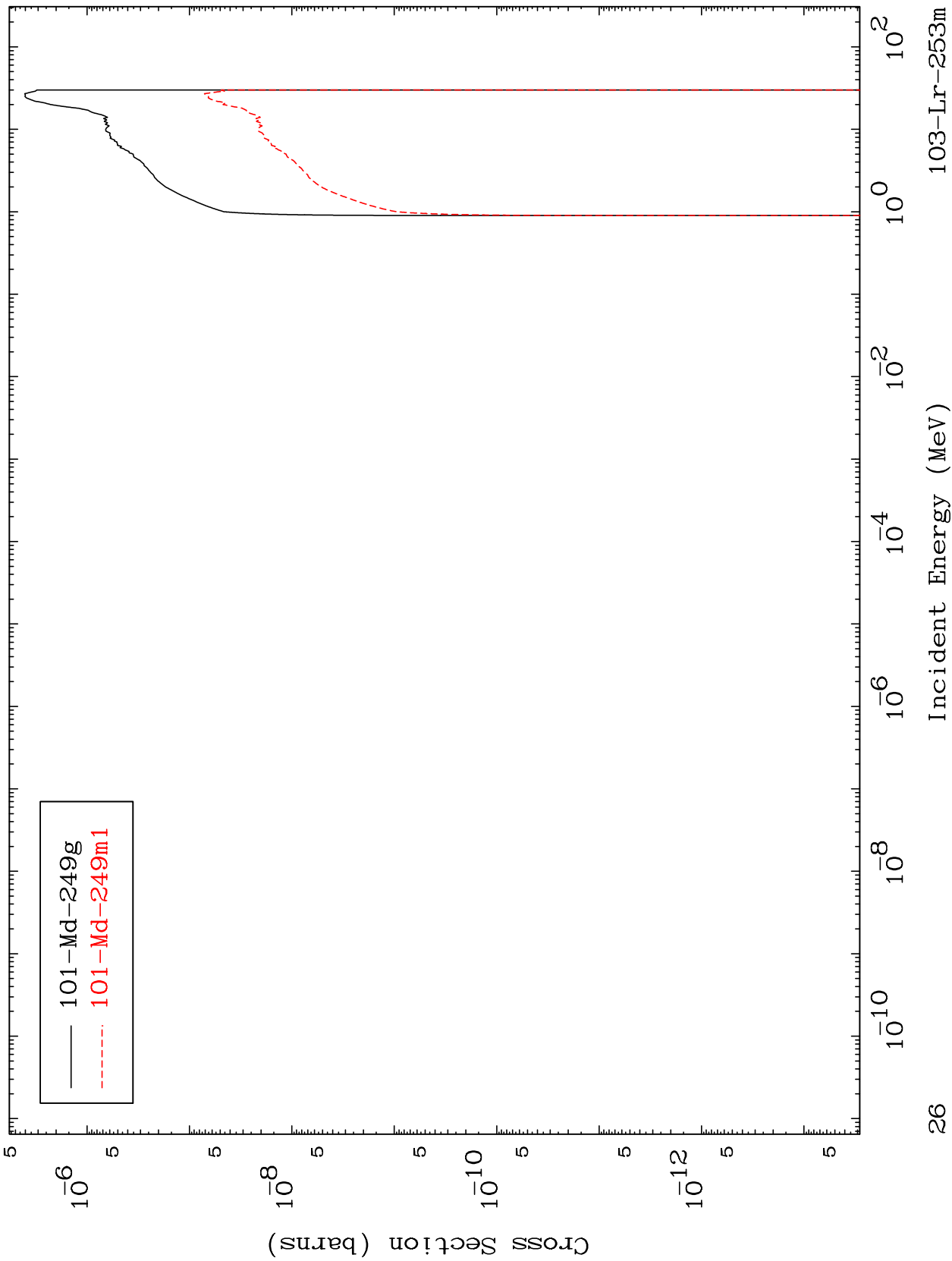
— 103-Lr-253g  
- - - 103-Lr-253m2

MAT 9987

$(n, n') \alpha$

$^{103}\text{Lr-253m}$

Radionuclide Production Cross Section

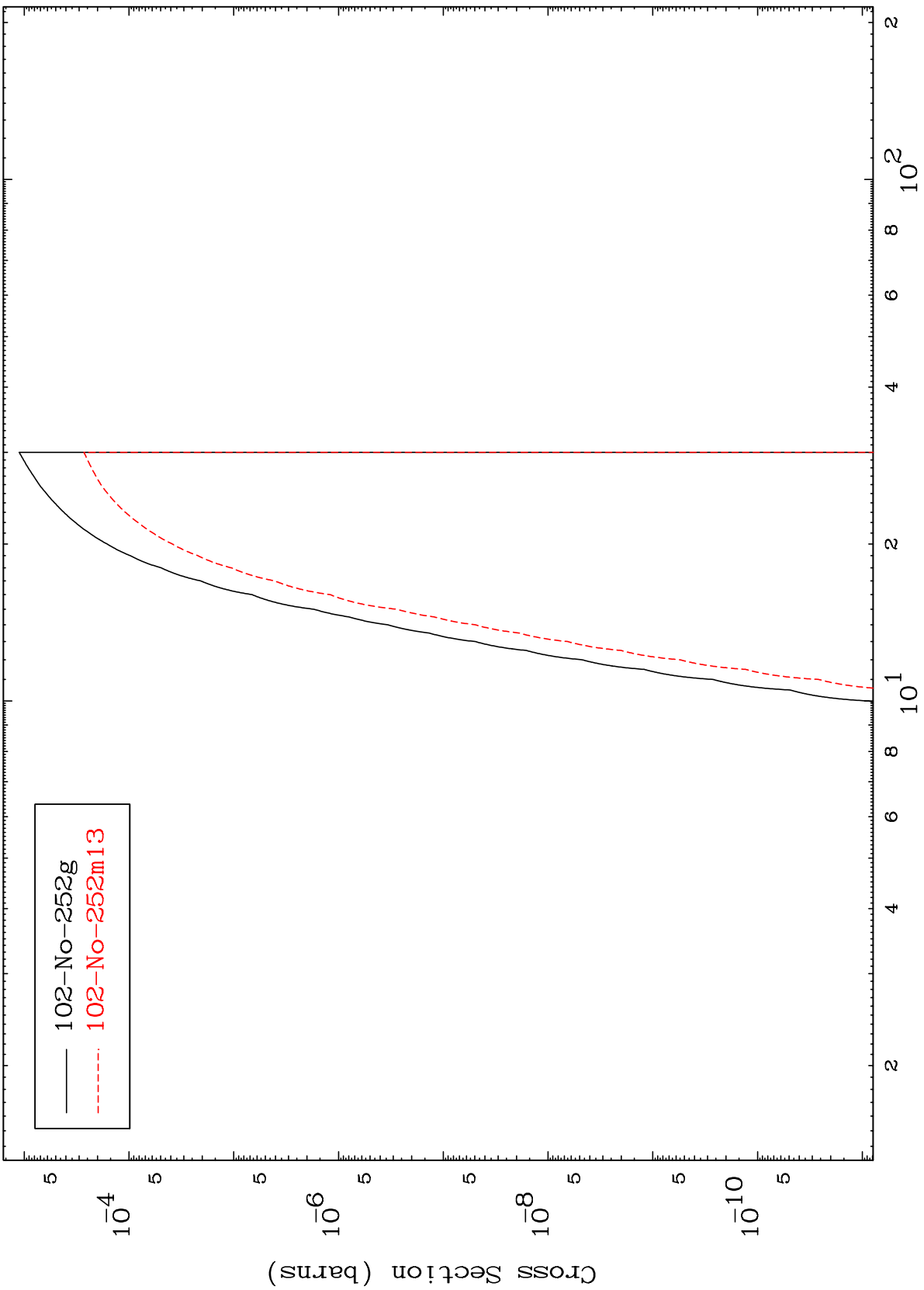


MAT 9987

(n,n') p

103-Lr-253m

Radionuclide Production Cross Section



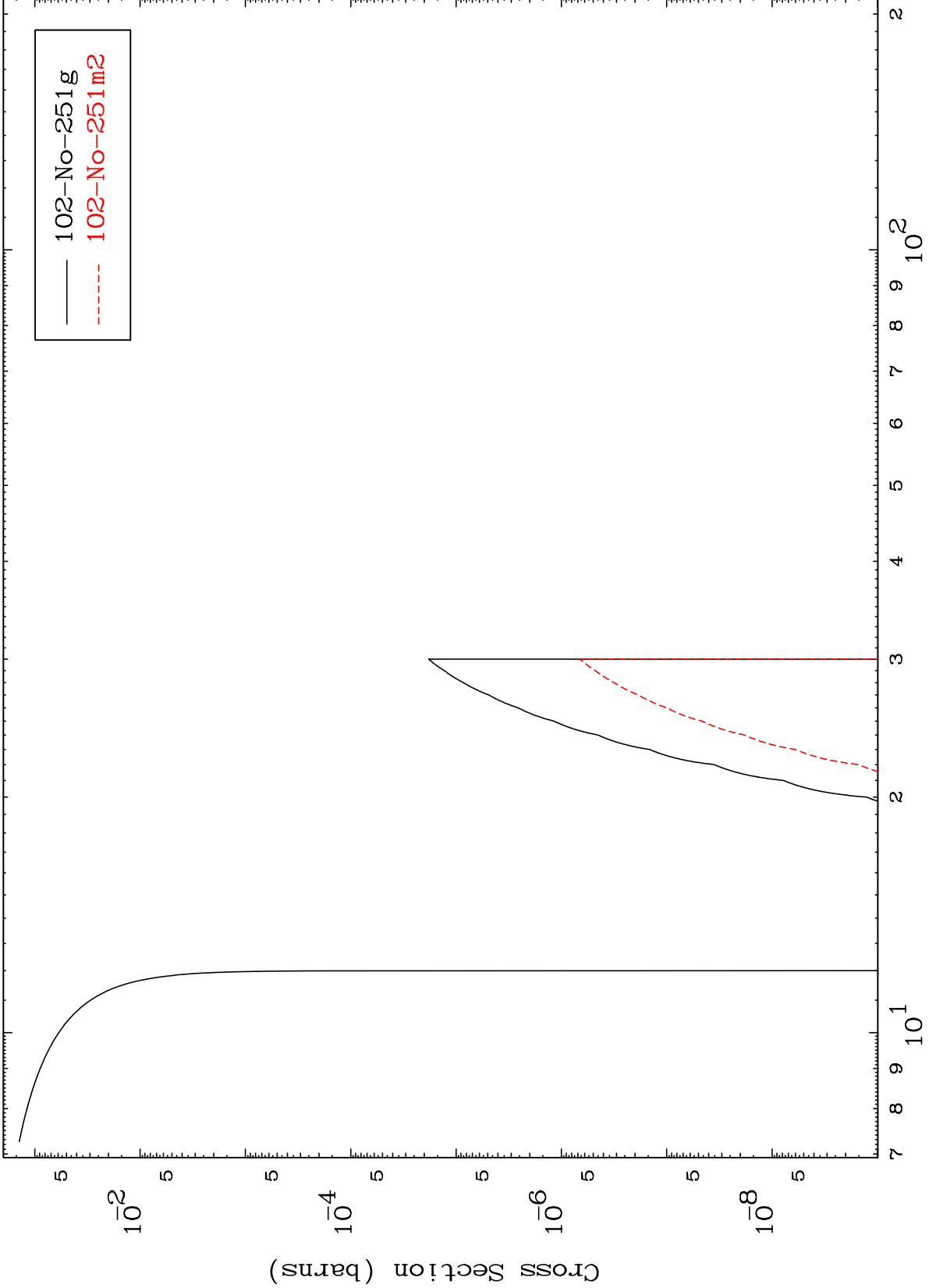
— 102-No-252g  
- - - 102-No-252m13

MAT 9987

(n,n') d

103-Lr-253m

Radionuclide Production Cross Section



28

Incident Energy (MeV)

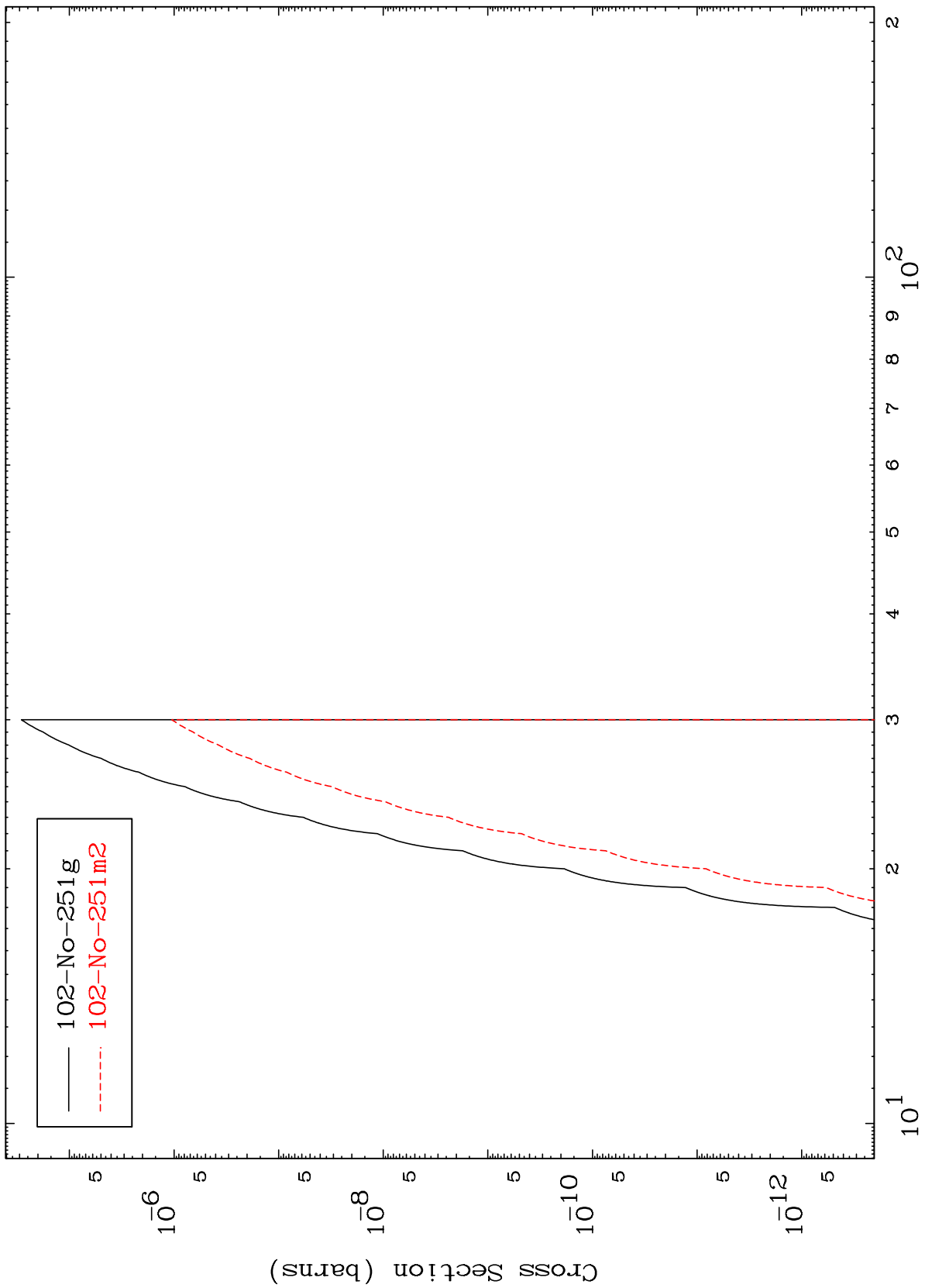
103-Lr-253m

MAT 9987

(n,2n) p

103-Lr-253m

Radionuclide Production Cross Section



29

Incident Energy (MeV)

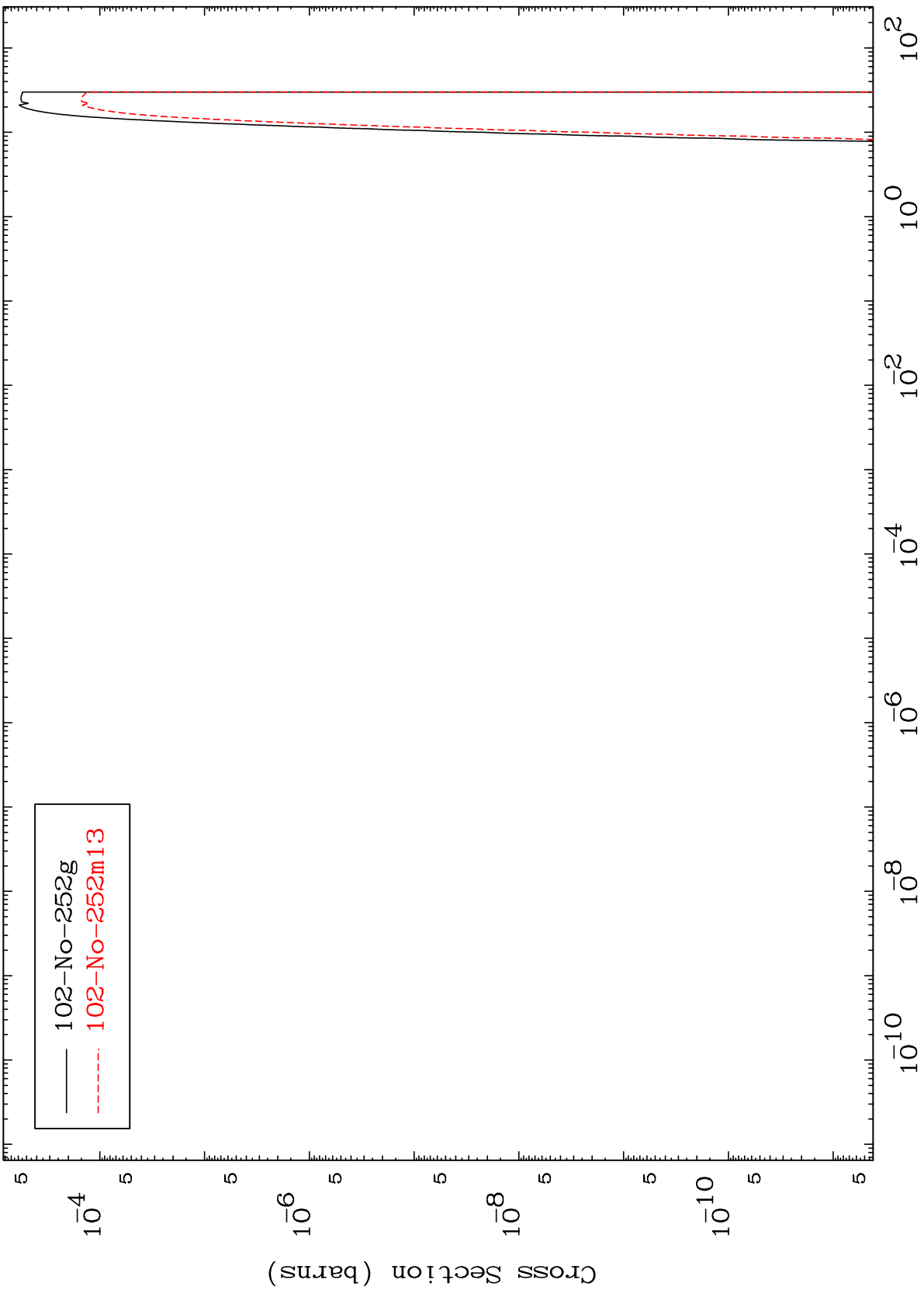
103-Lr-253m

MAT 9987

(n,d)

103-Lr-253m

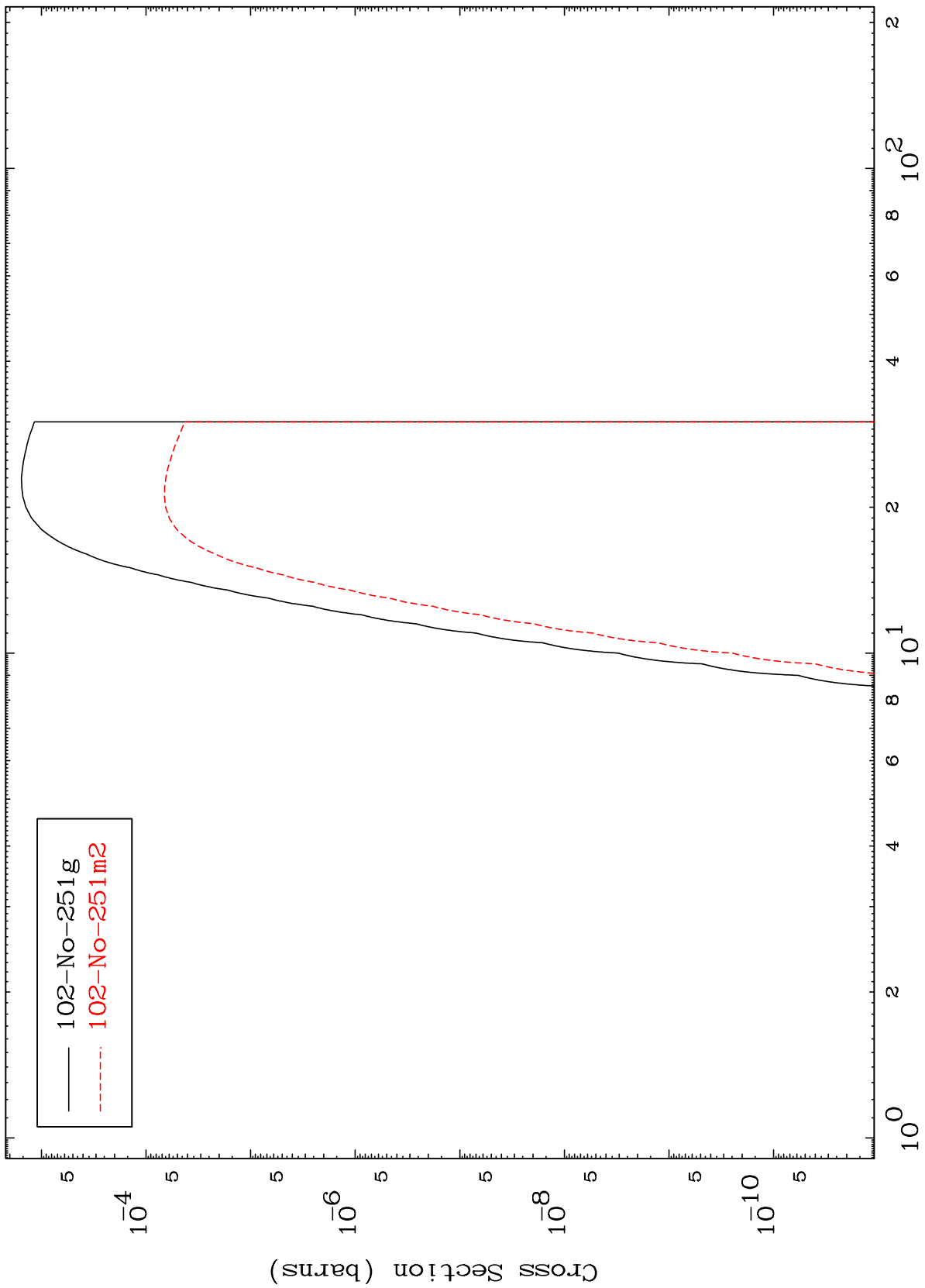
Radionuclide Production Cross Section



MAT 9987

103-Lr-253m

(n, t)  
Radionuclide Production Cross Section



103-Lr-253m

Incident Energy (MeV)

31