

Program EVALPLOT  
(Version 2021-1)

by

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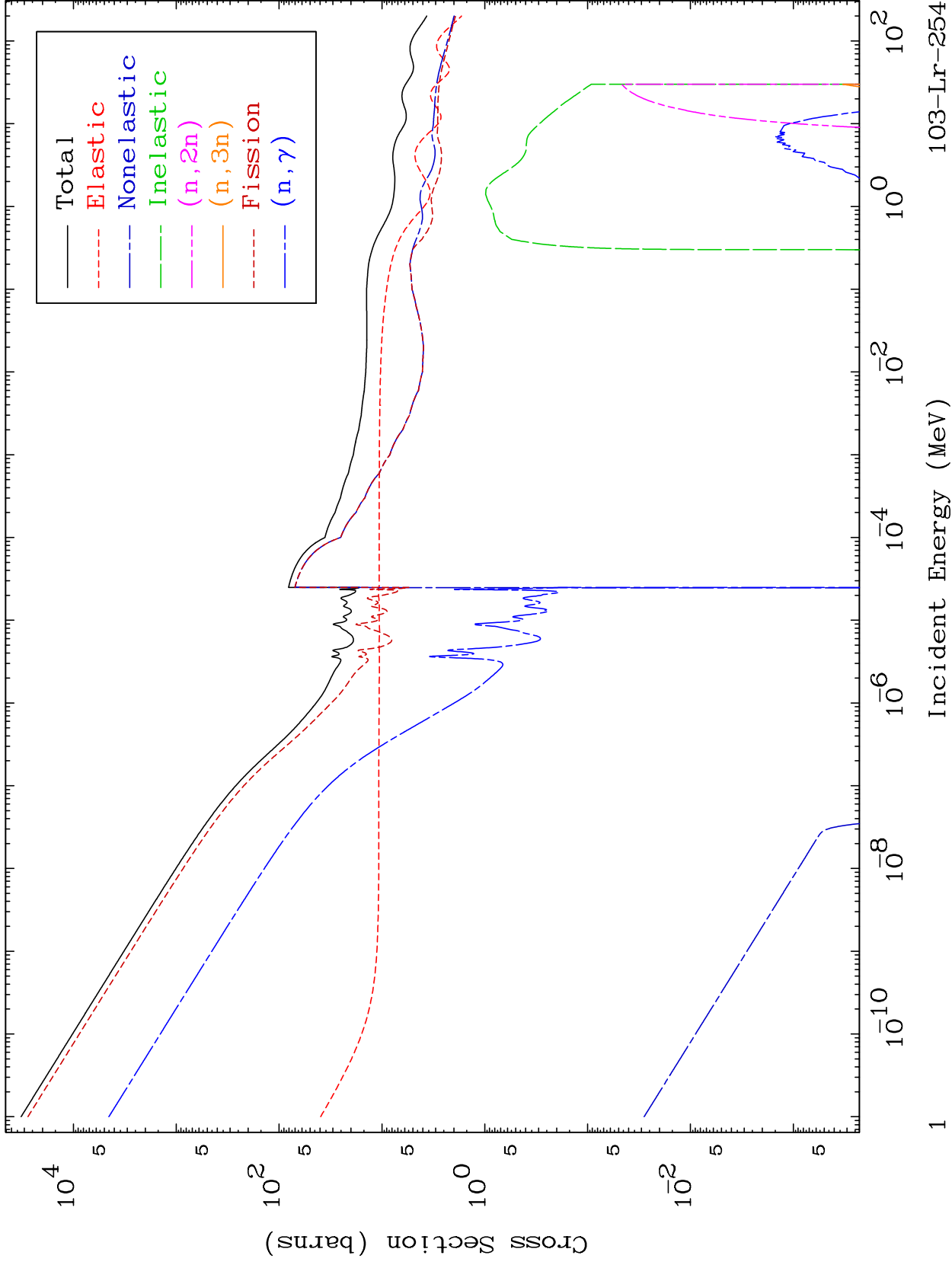
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9983

Neutron Major  
293 Kelvin Cross Sections

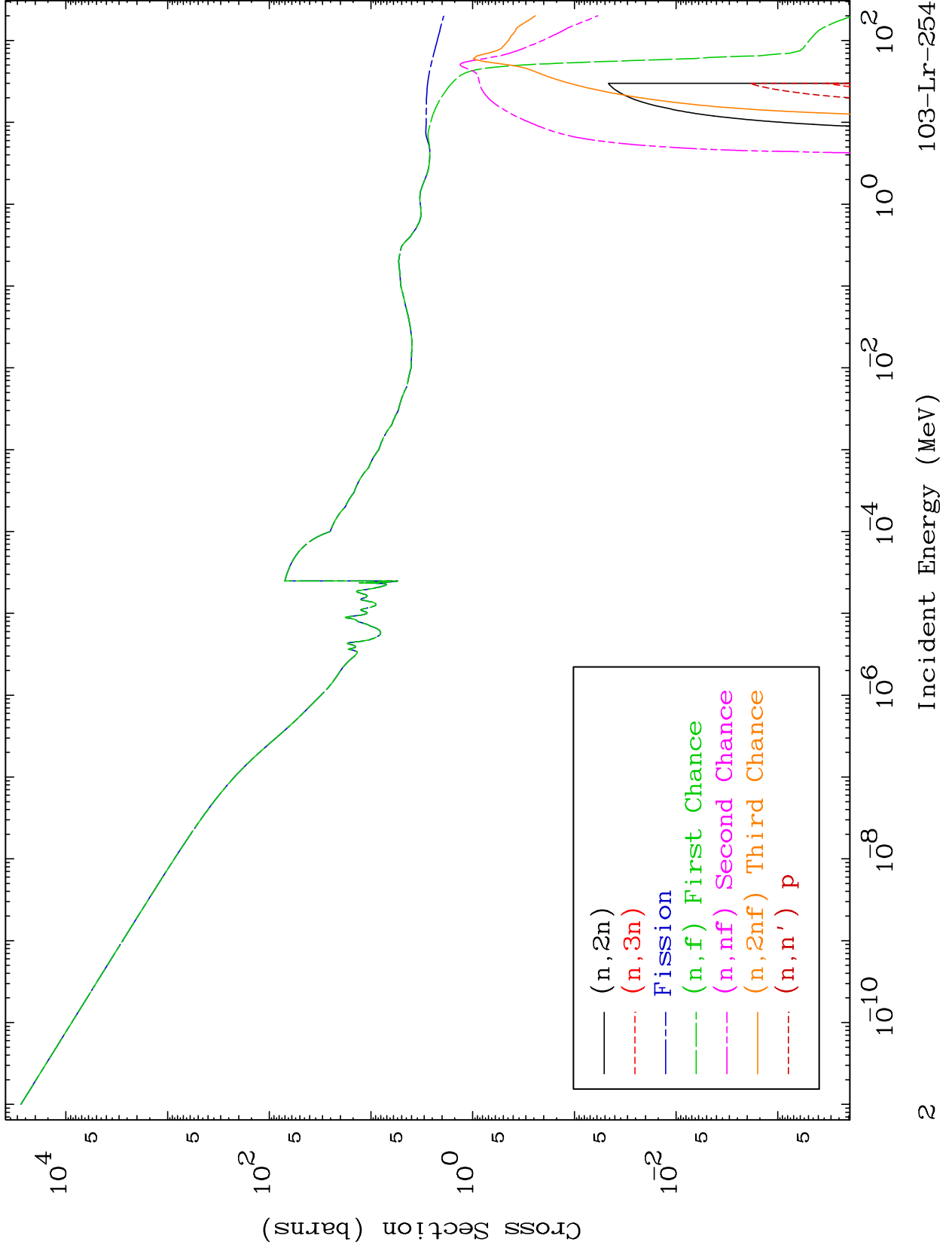
103-Lr-254

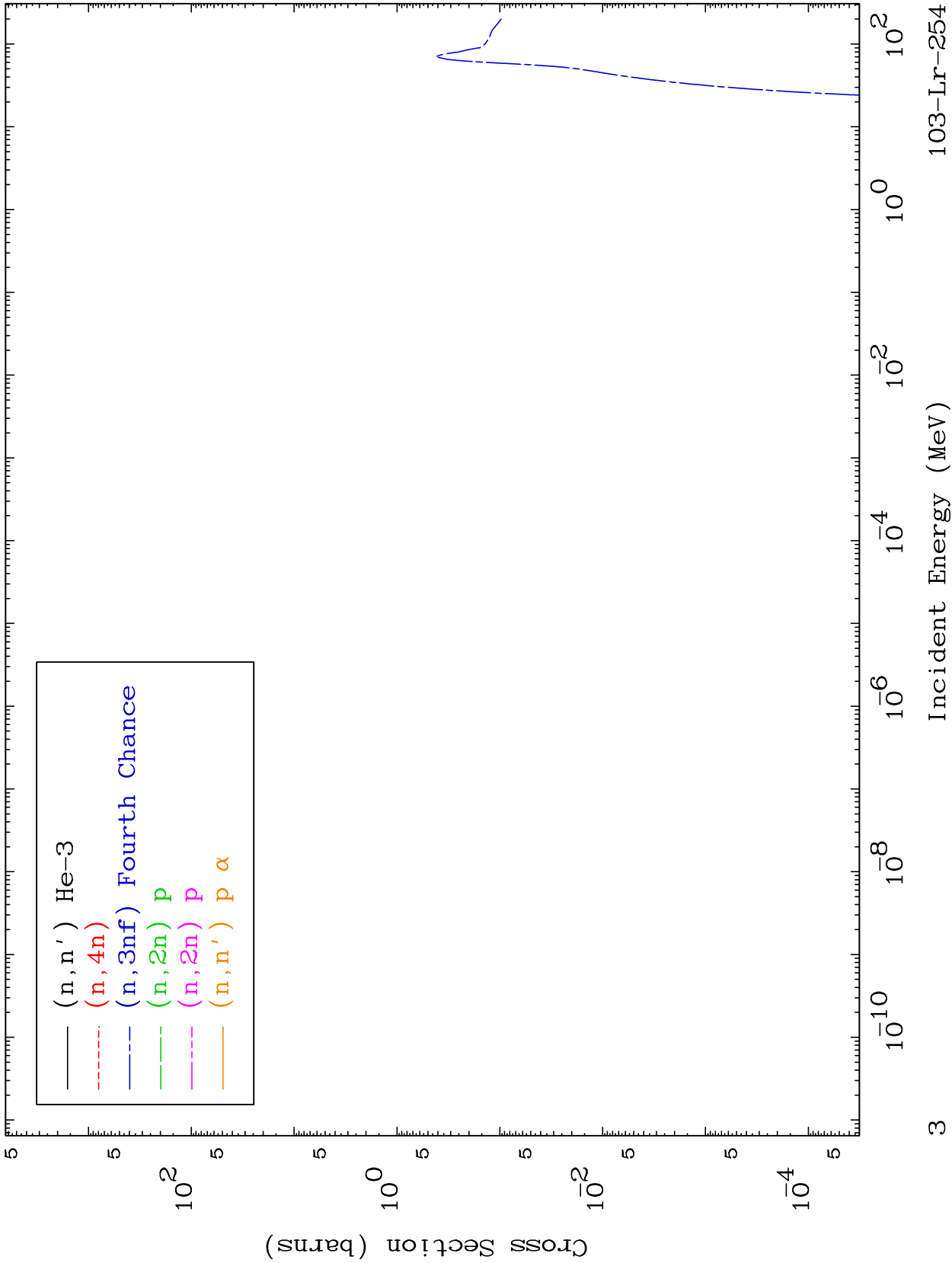


MAT 9983

Neutron Absorption  
293 Kelvin Cross Sections

103-Lr-254

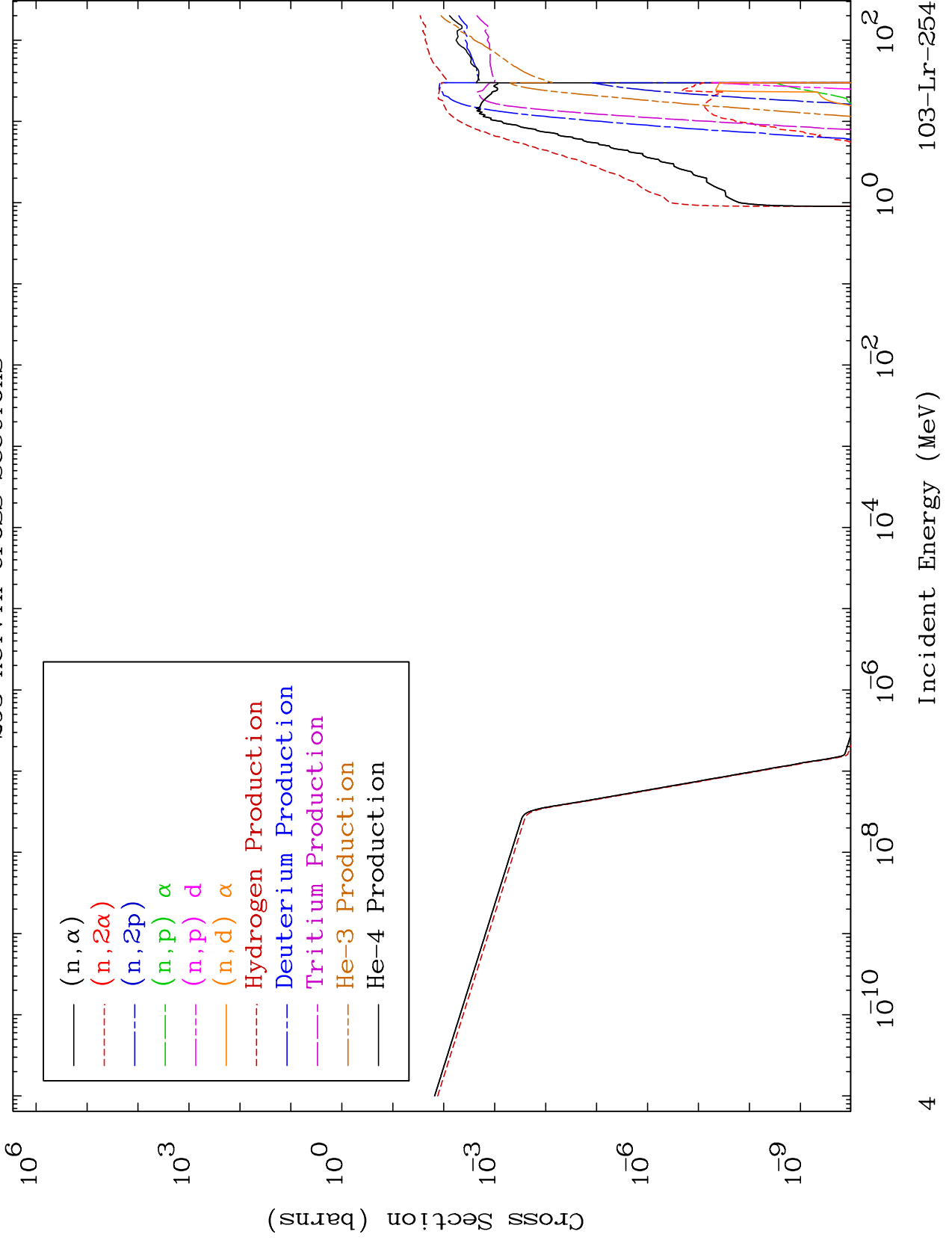




MAT 9983

Neutron Absorption  
293 Kelvin Cross Sections

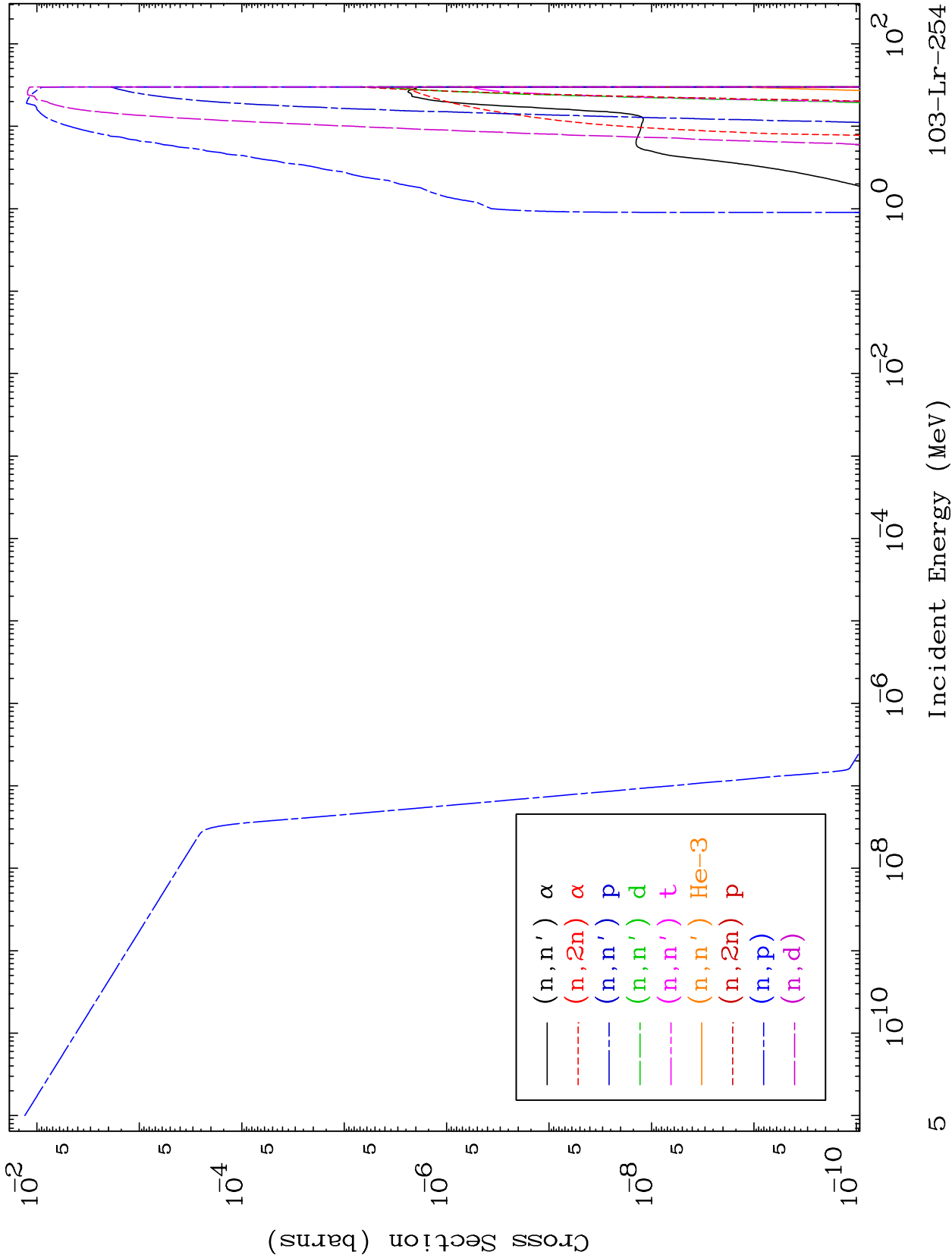
103-Lr-254



MAT 9983

Charged Particle  
293 Kelvin Cross Sections

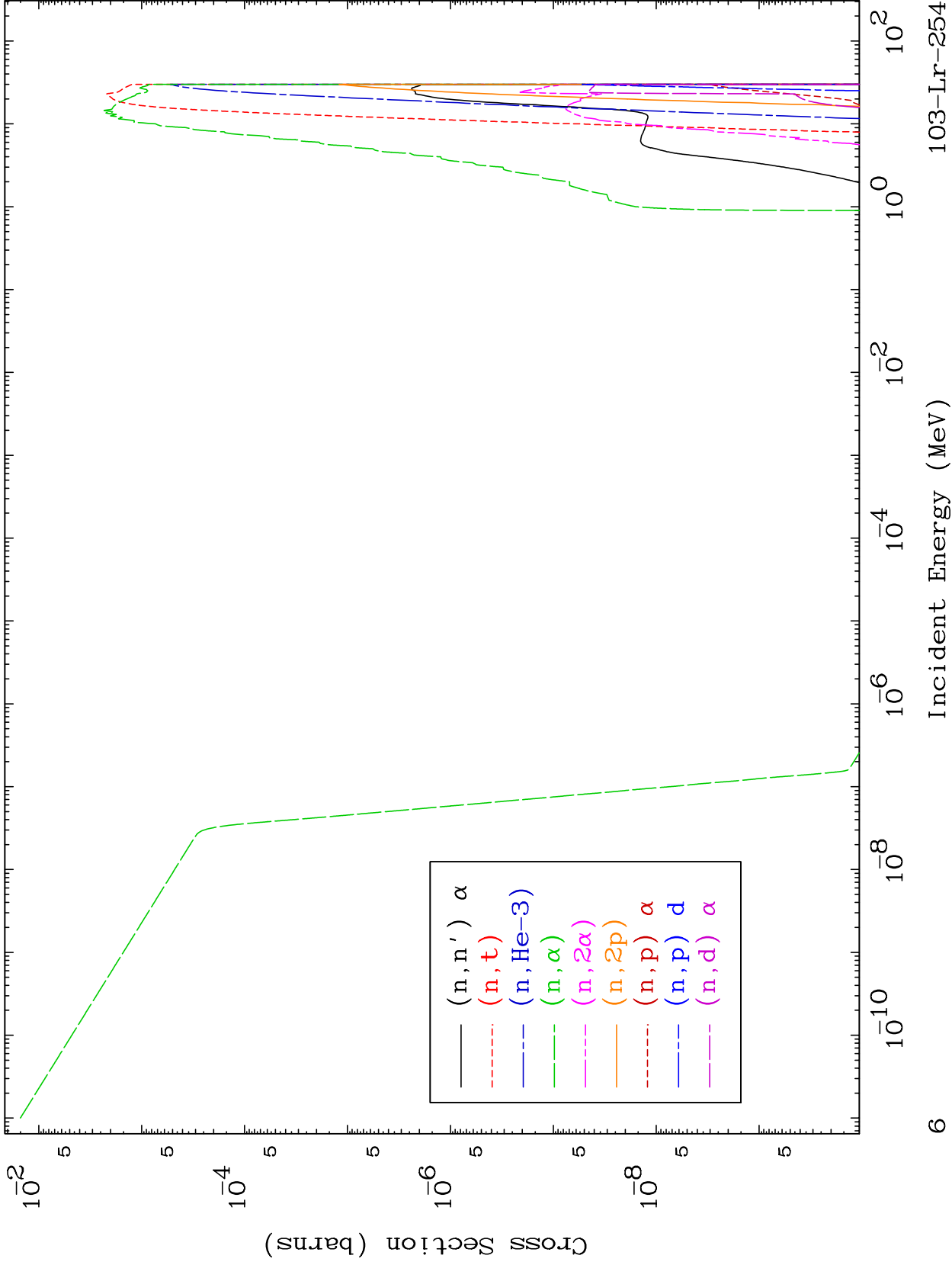
103-Lr-254



MAT 9983

Charged Particle  
293 Kelvin Cross Sections

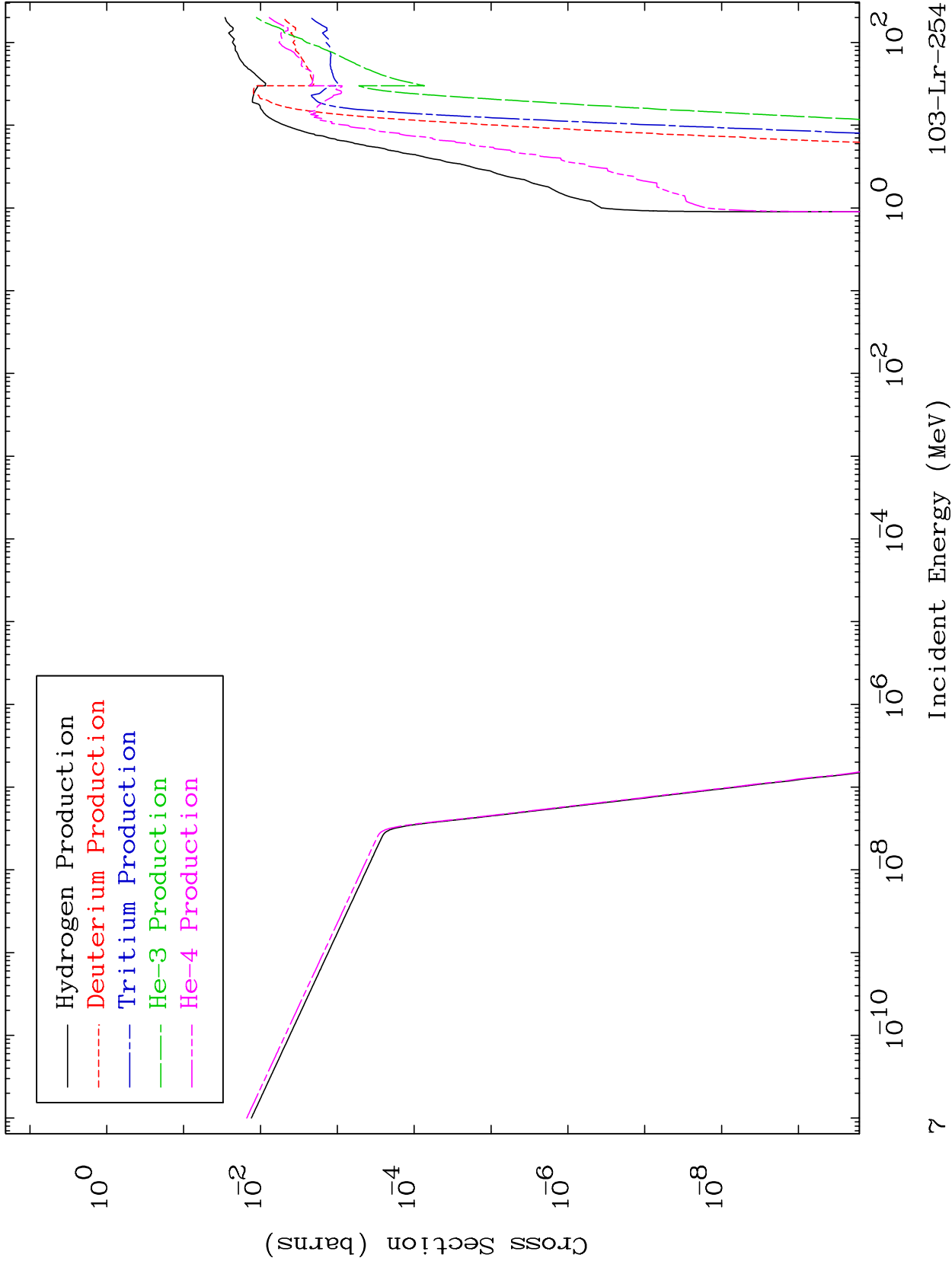
103-Lr-254



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Particle Production  
293 Kelvin Cross Sections

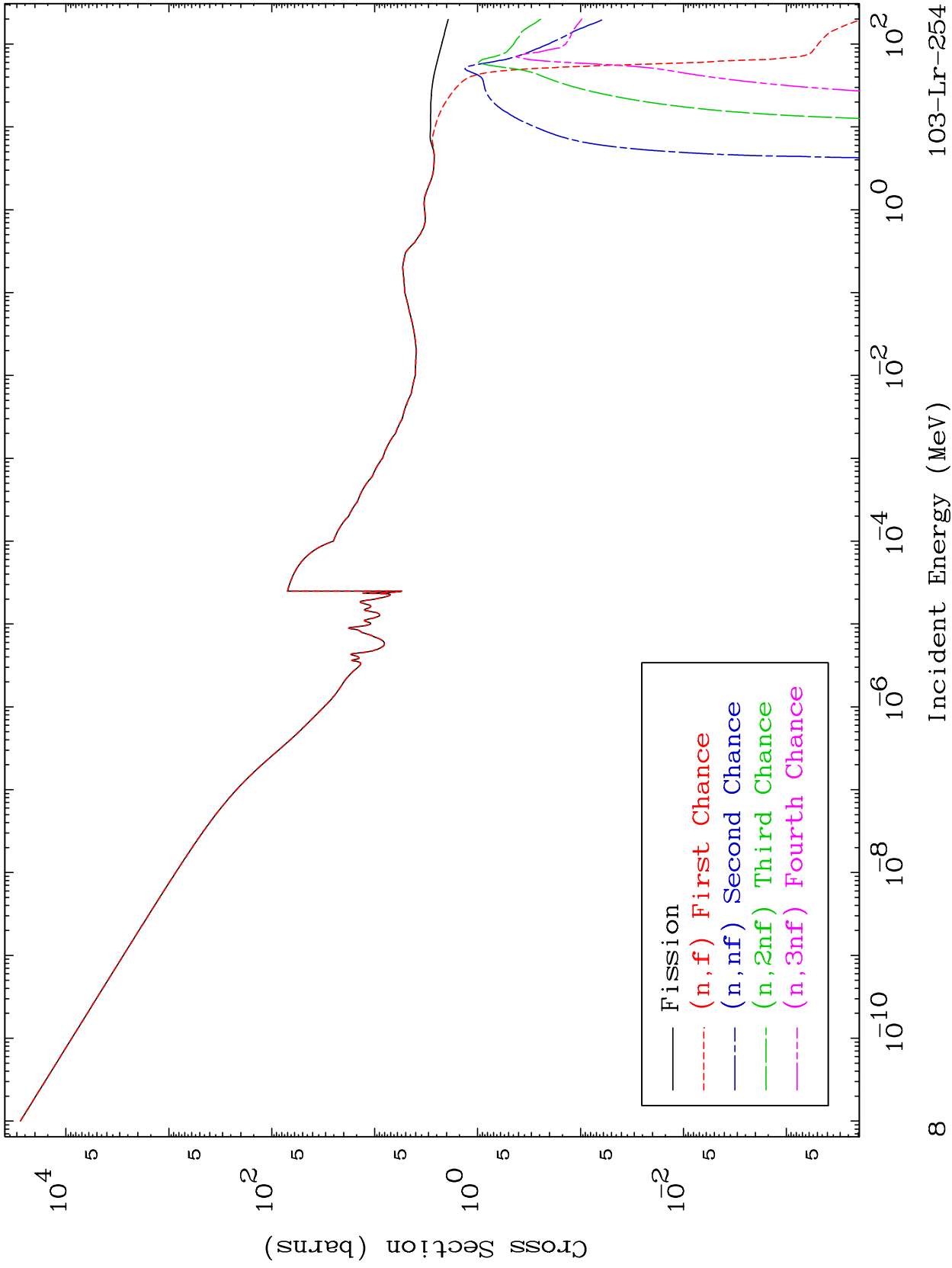
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MAT 9983

Fission  
293 Kelvin Cross Sections

103-Lr-254

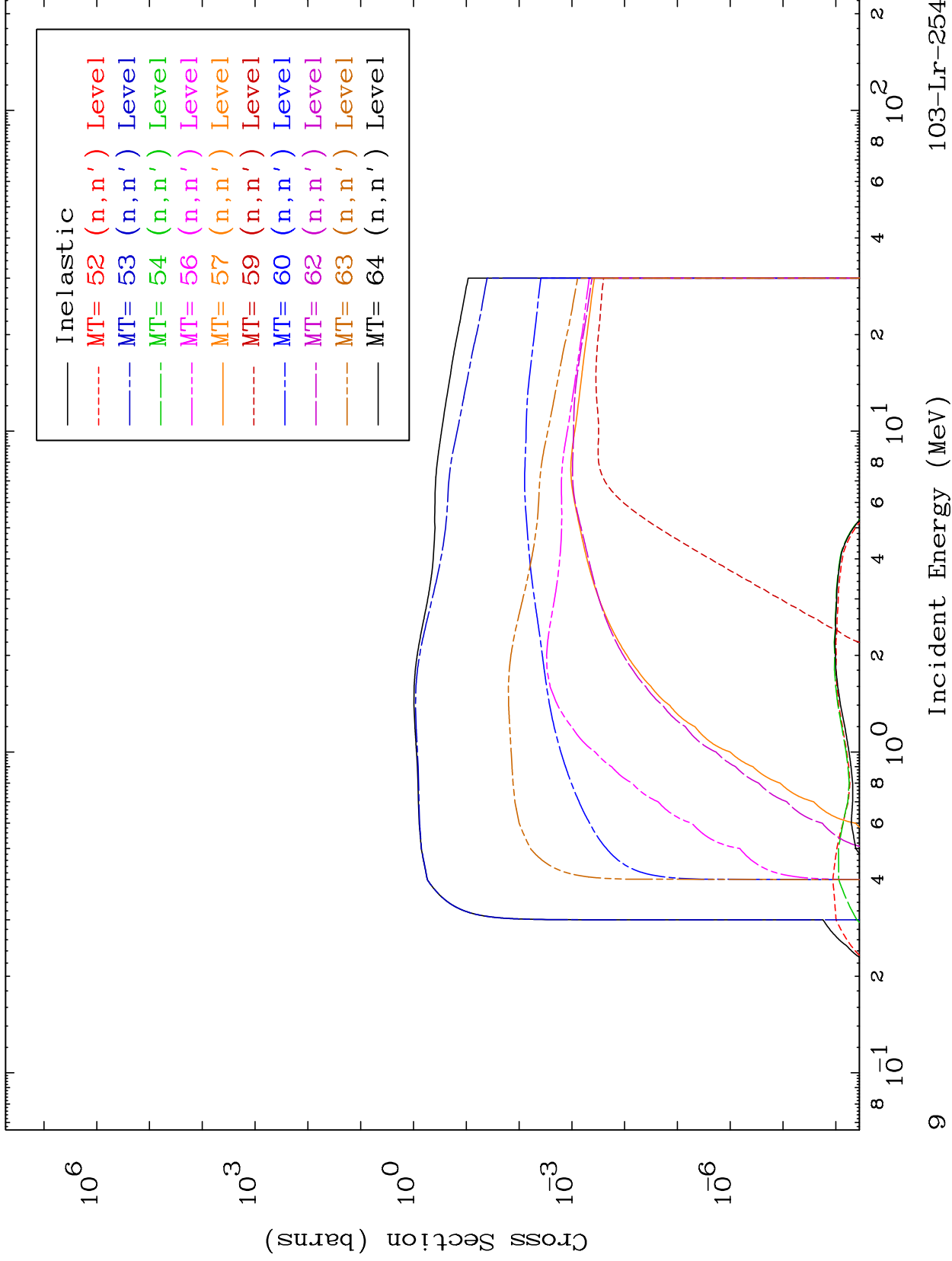


MAT 9983

(n,n') Levels

103-Lr-254

293 Kelvin Cross Sections



9

Incident Energy (MeV)

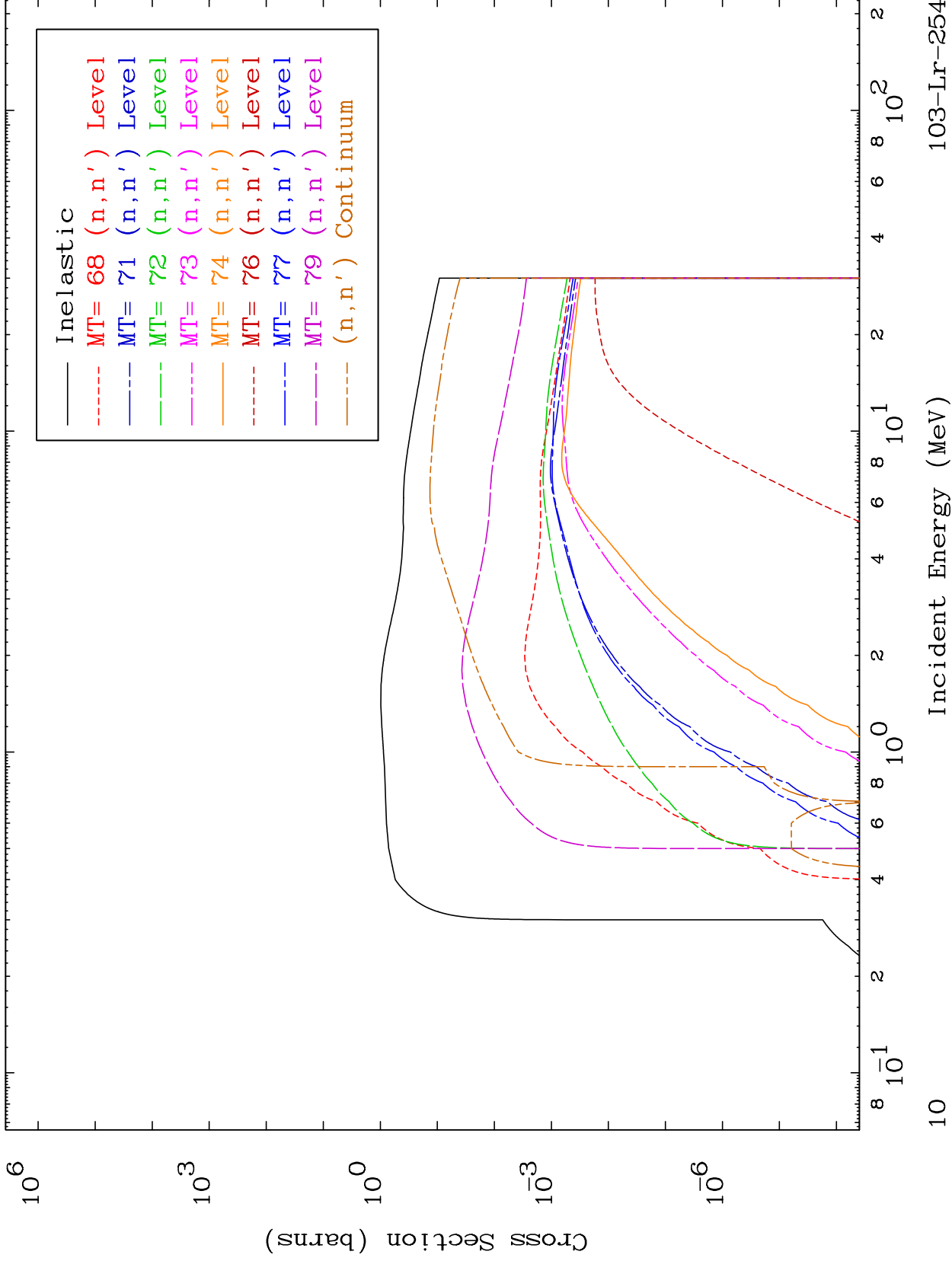
103-Lr-254

MAT 9983

(n,n') Levels

103-Lr-254

293 Kelvin Cross Sections



10

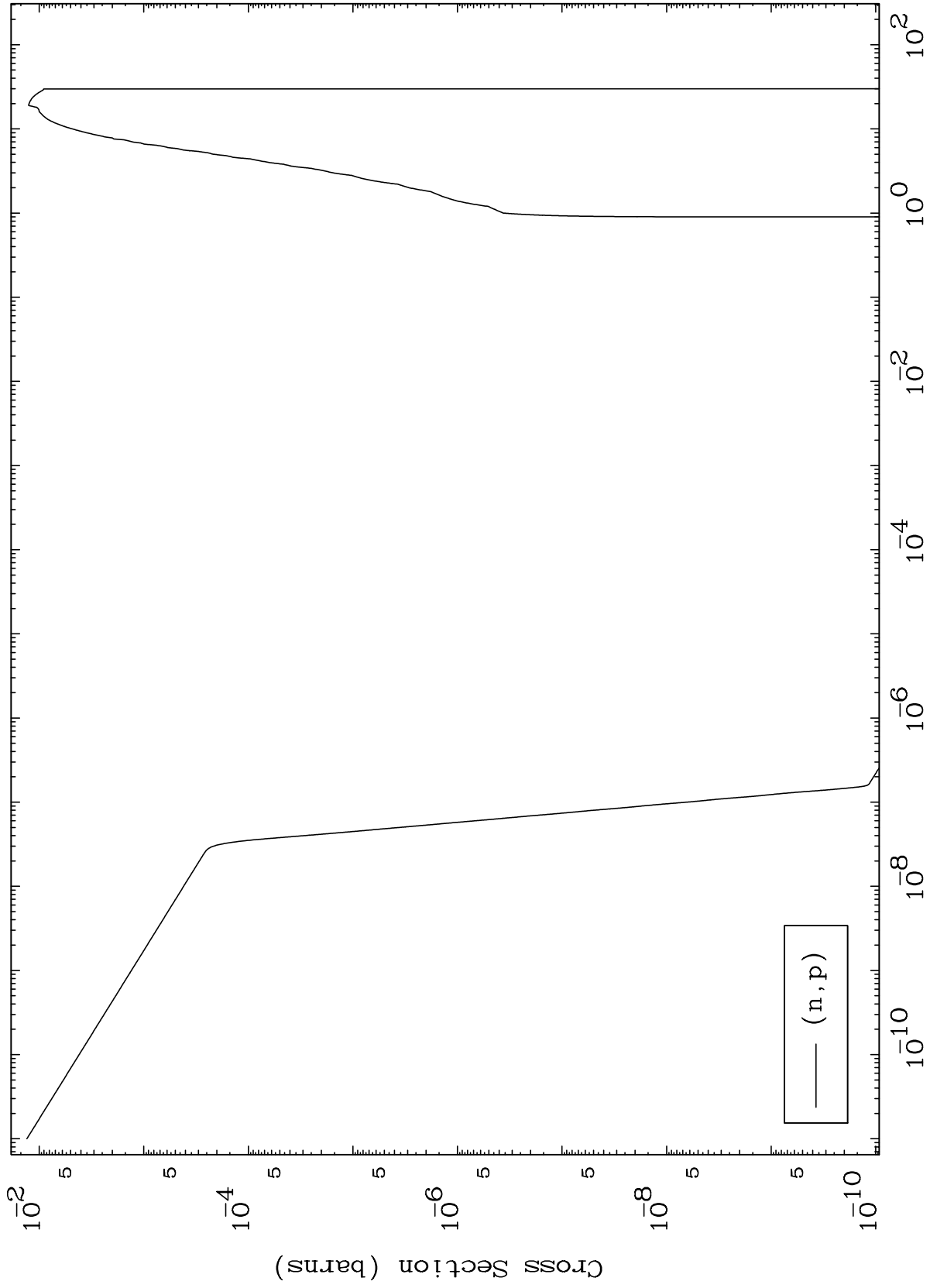
Incident Energy (MeV)

103-Lr-254

MAT 9983

(n,p) Levels  
293 Kelvin Cross Sections

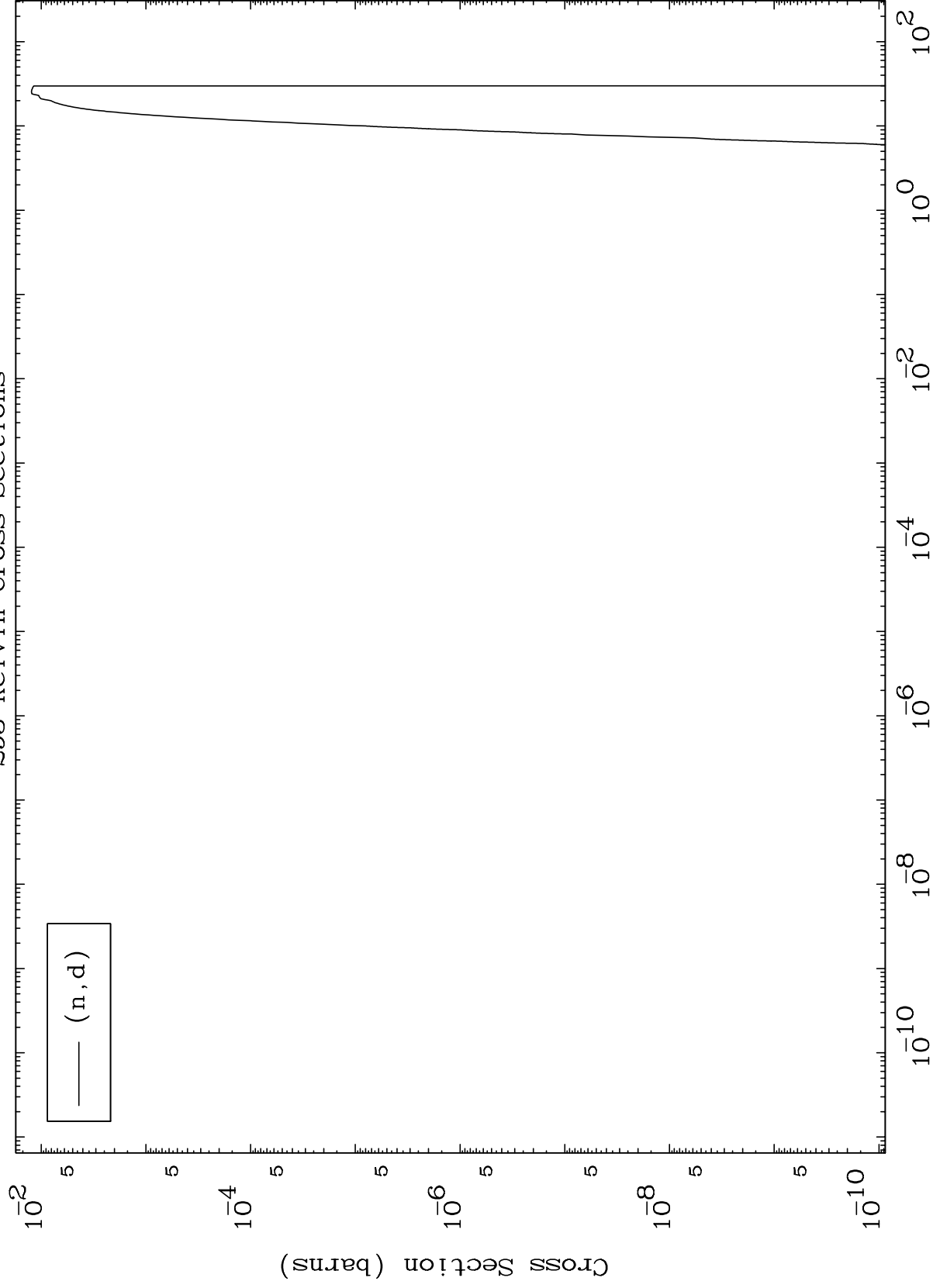
103-Lr-254



MAT 9983

(n,d) Levels  
293 Kelvin Cross Sections

103-Lr-254



12

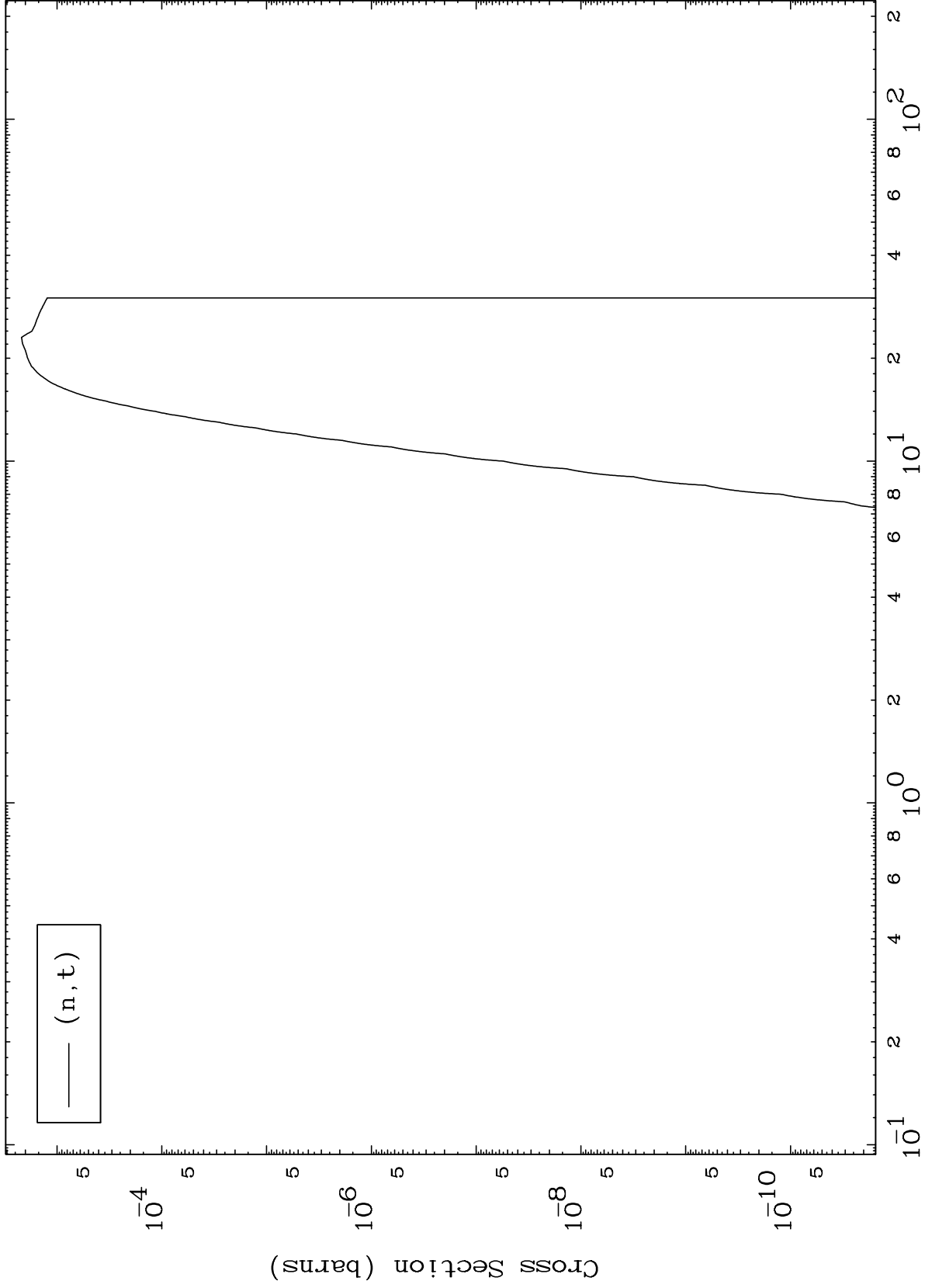
Incident Energy (MeV)

103-Lr-254

MAT 9983

(n,t) Levels  
293 Kelvin Cross Sections

103-Lr-254



13

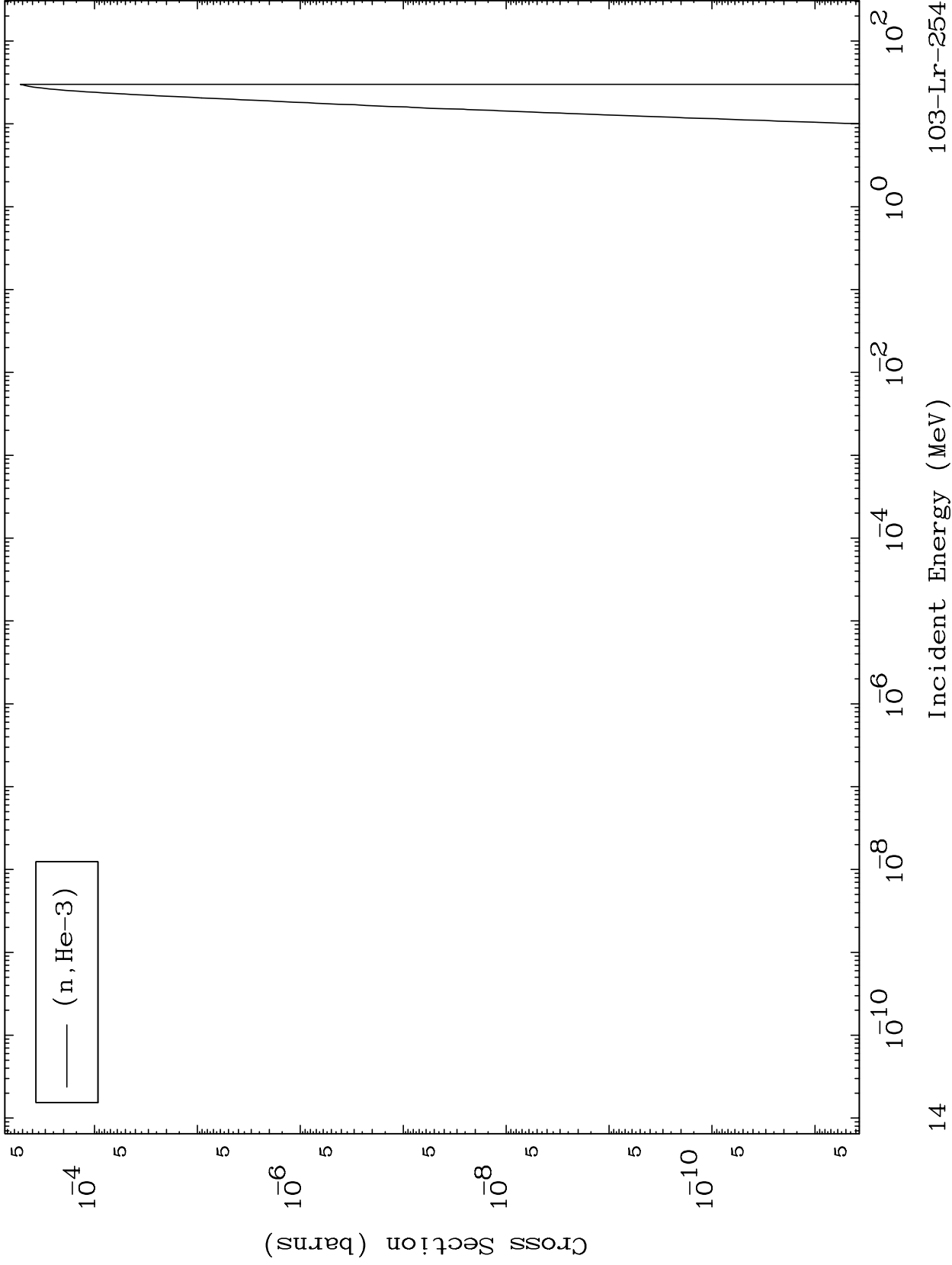
Incident Energy (MeV)

103-Lr-254

MAT 9983

(n,He3) Levels  
293 Kelvin Cross Sections

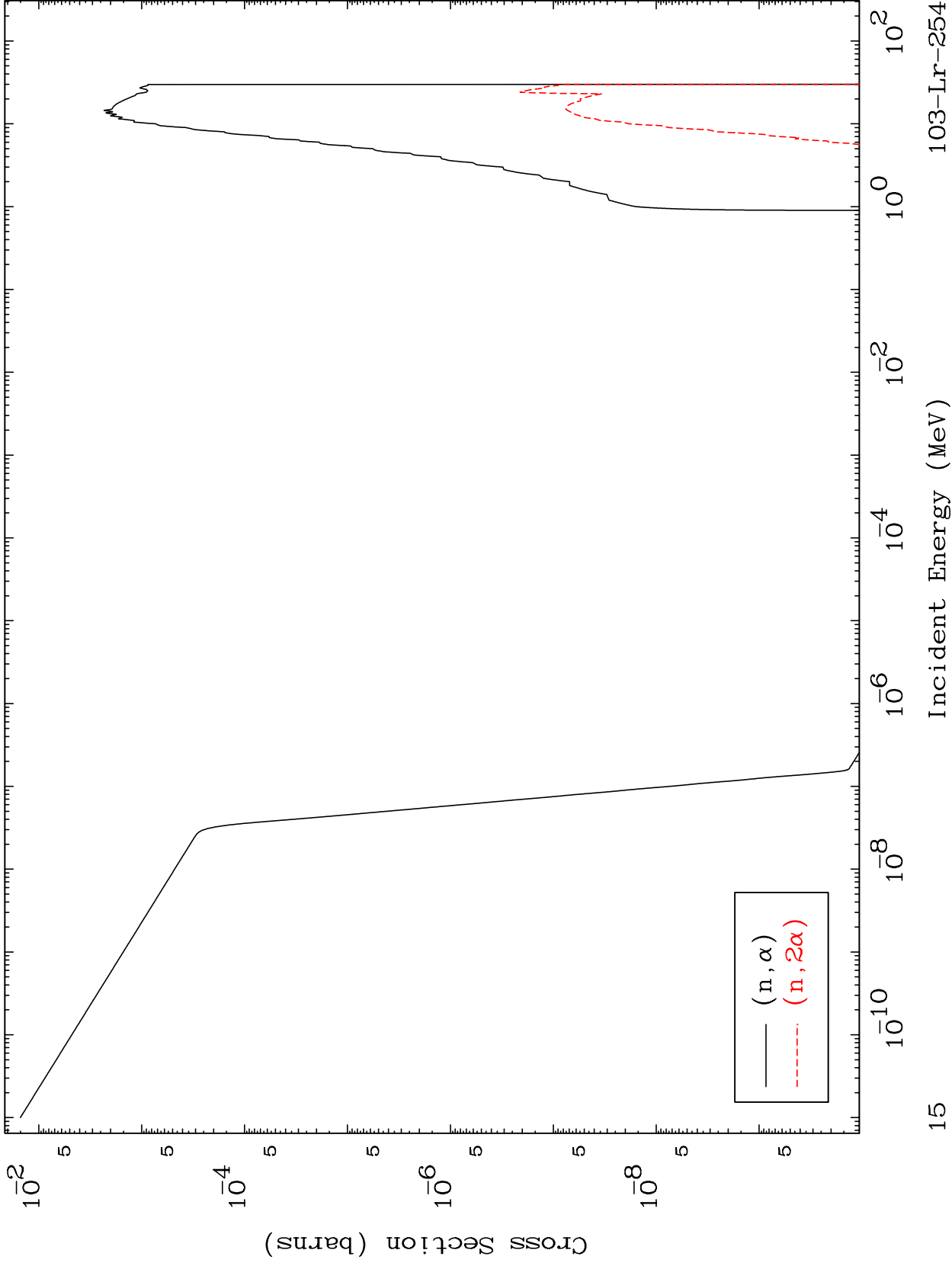
103-Lr-254



MAT 9983

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

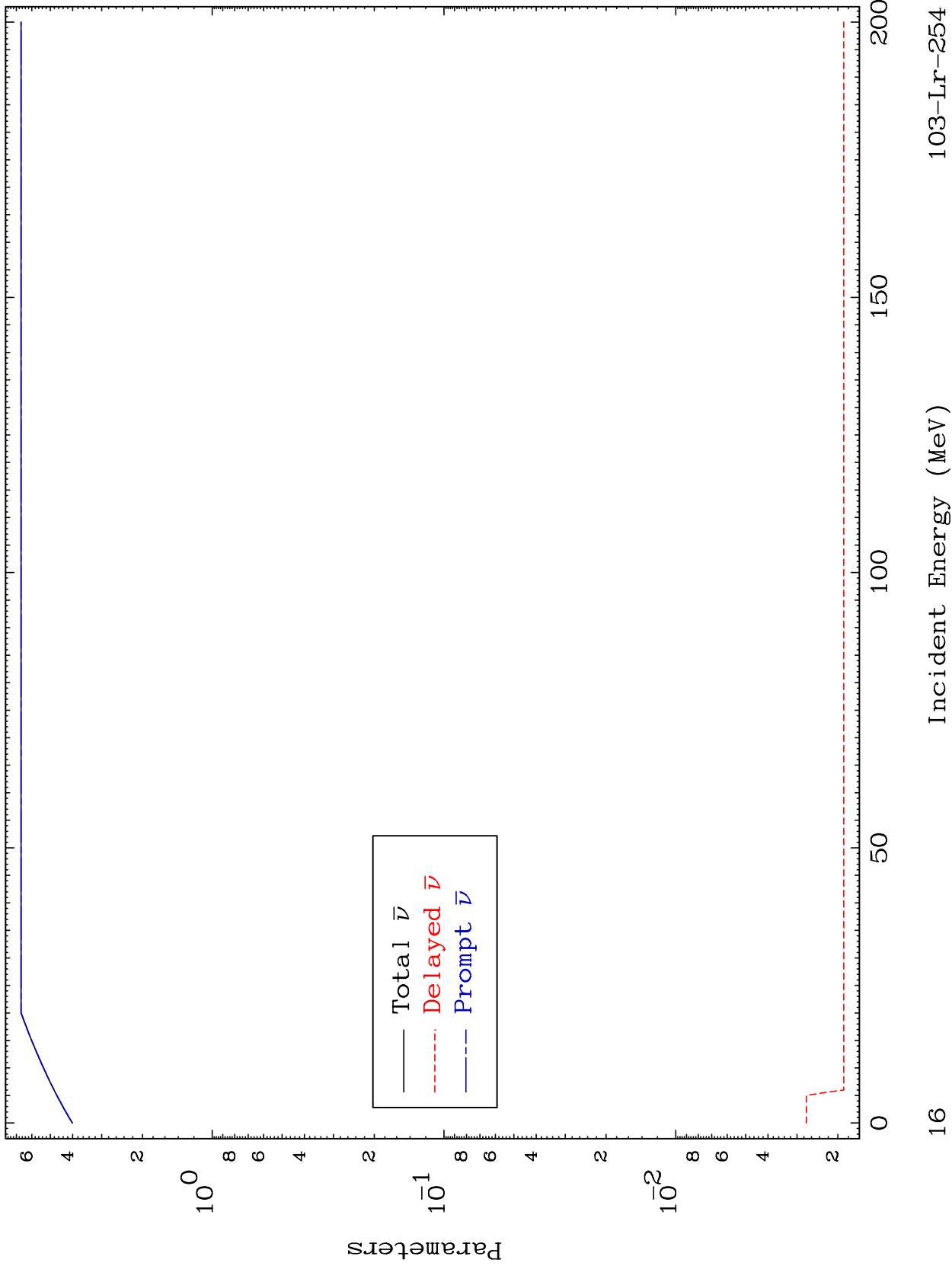
103-Lr-254



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Energy Release  
Parameters

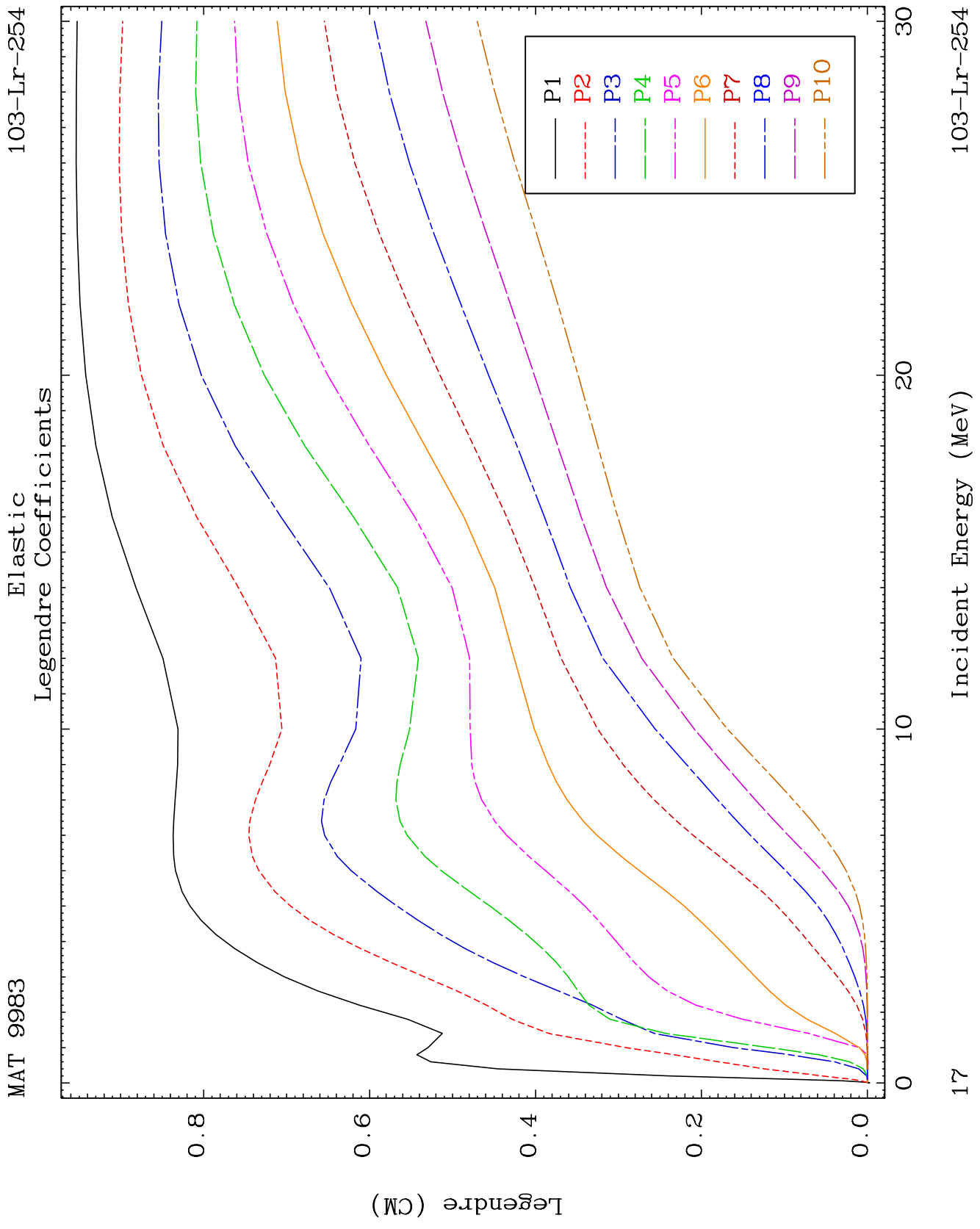
103-Lr-254



16

Incident Energy (MeV)

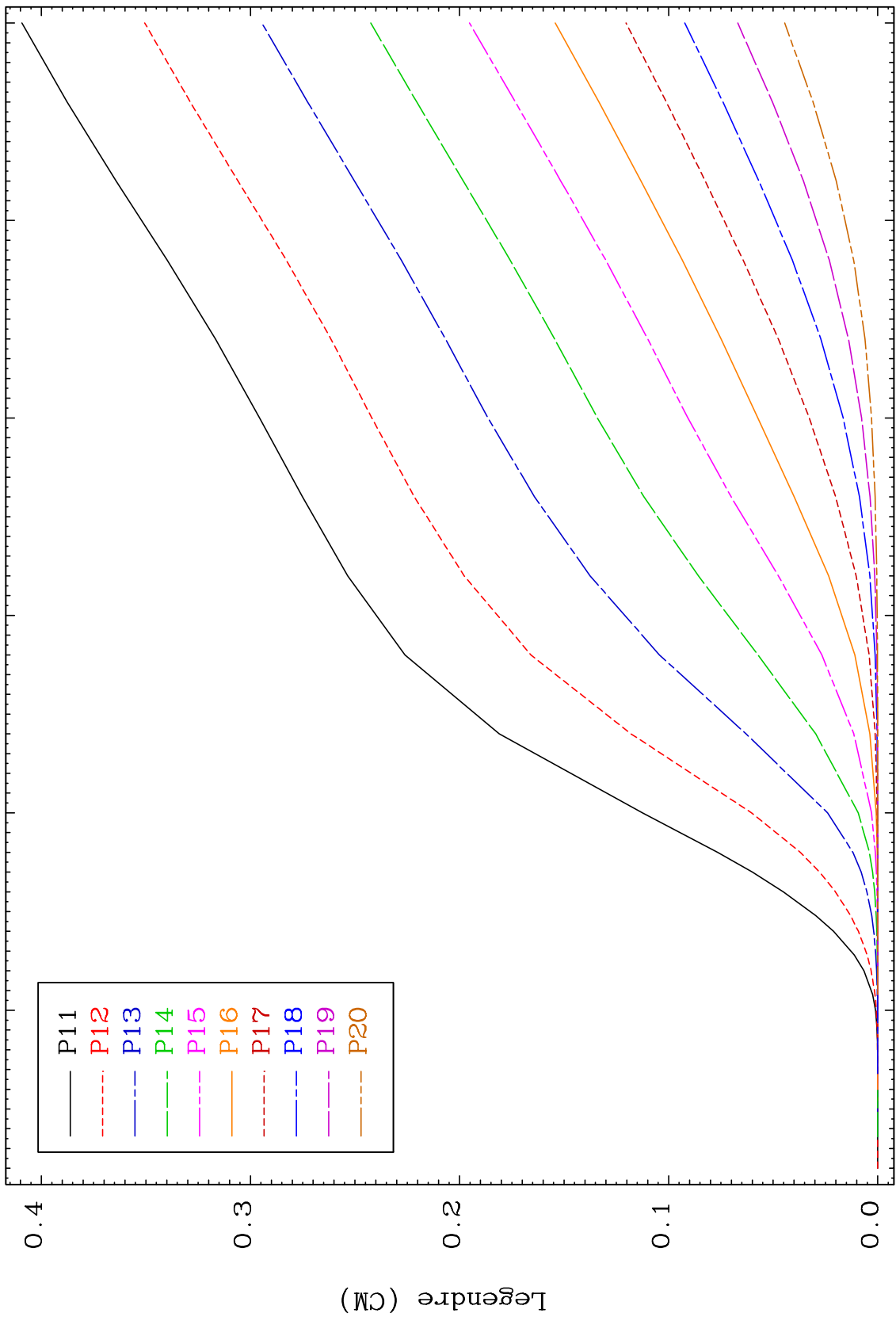
103-Lr-254



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Elastic Legendre Coefficients

103-Lr-254



18

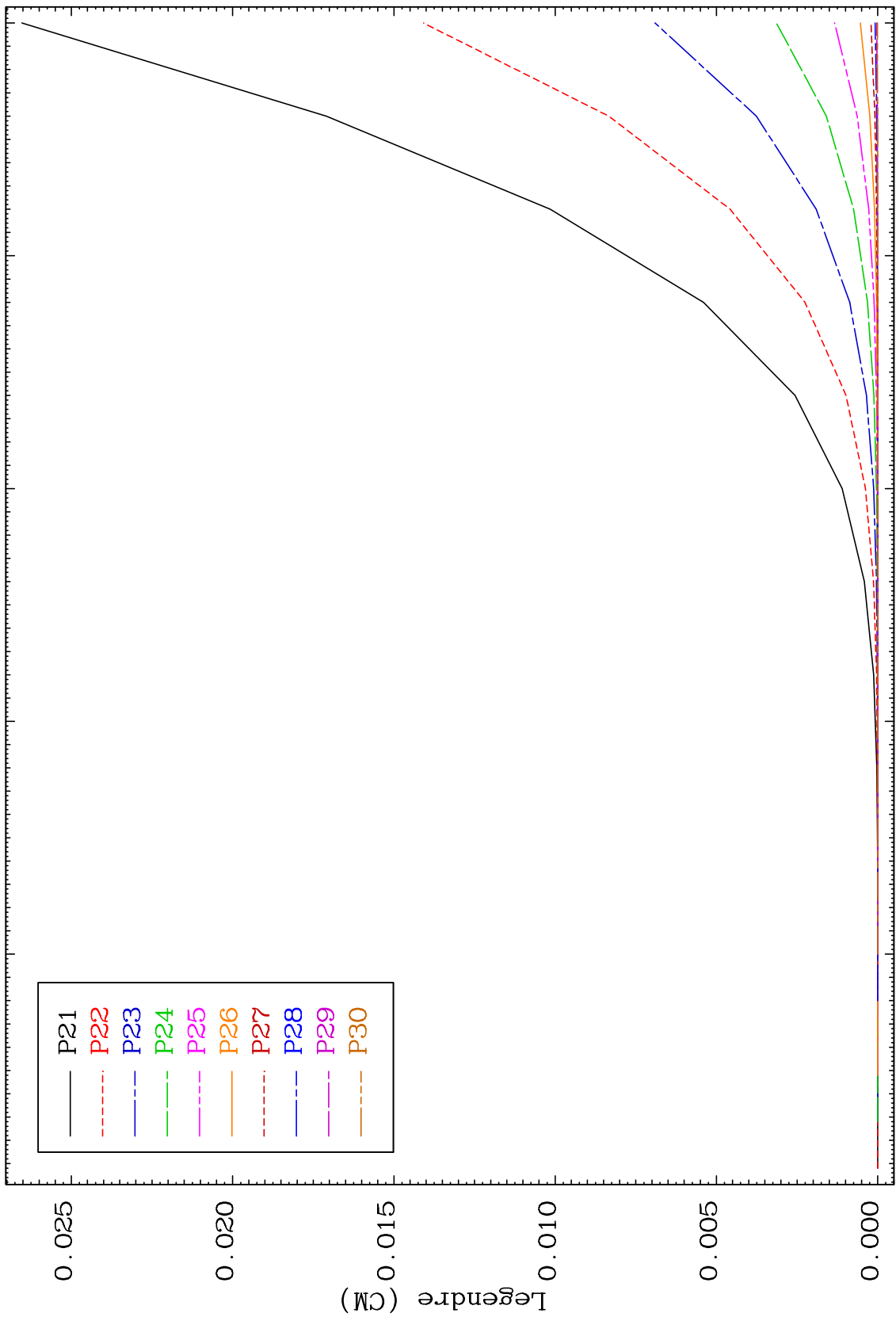
Incident Energy (MeV)

103-Lr-254

MAT 9983

Elastic Legendre Coefficients

103-Lr-254



19

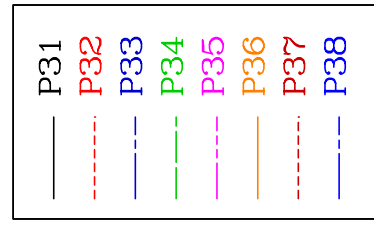
Incident Energy (MeV)

103-Lr-254

MAT 9983

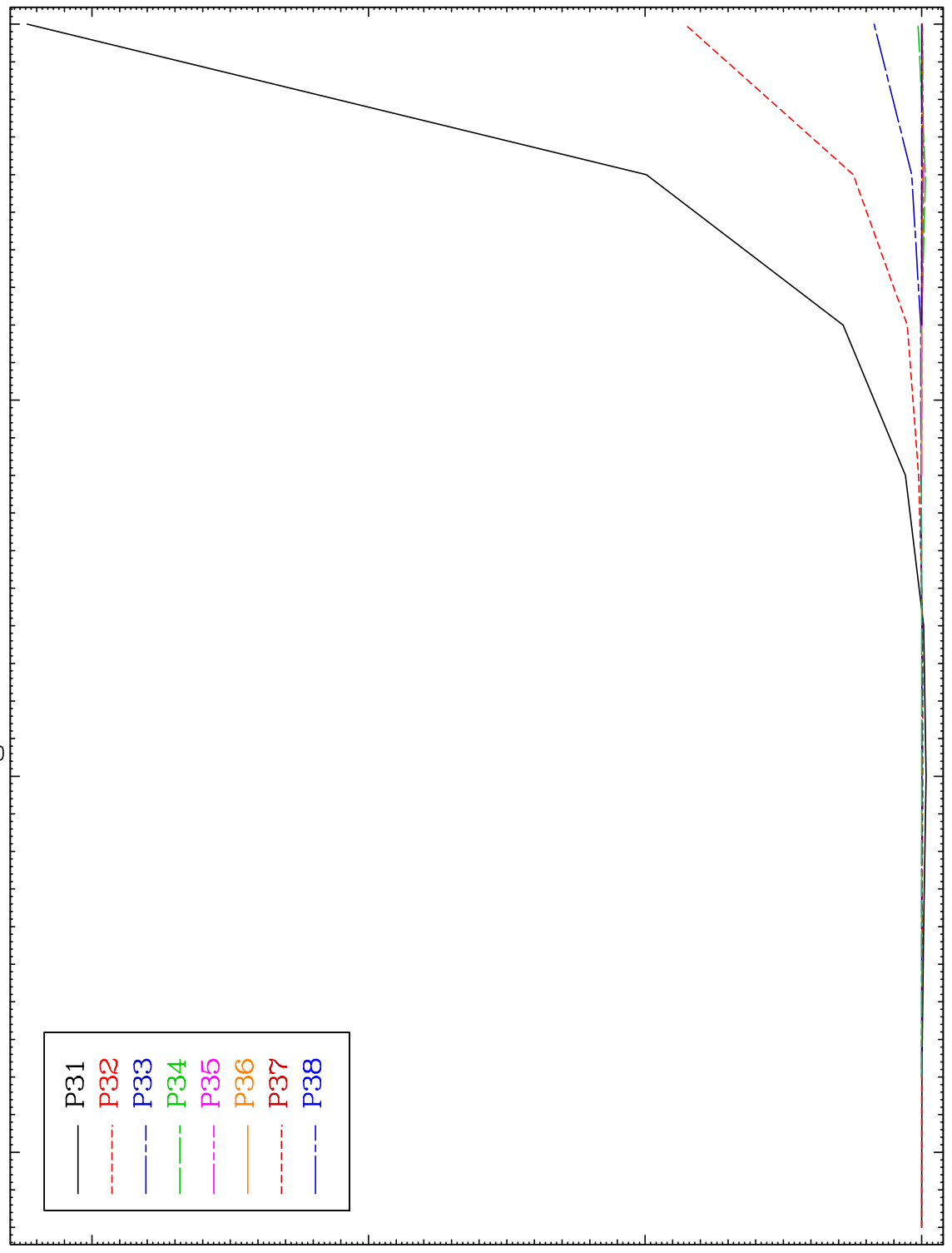
Elastic Legendre Coefficients

103-Lr-254



$\times 10^{-6}$

Legendre (CM)



20

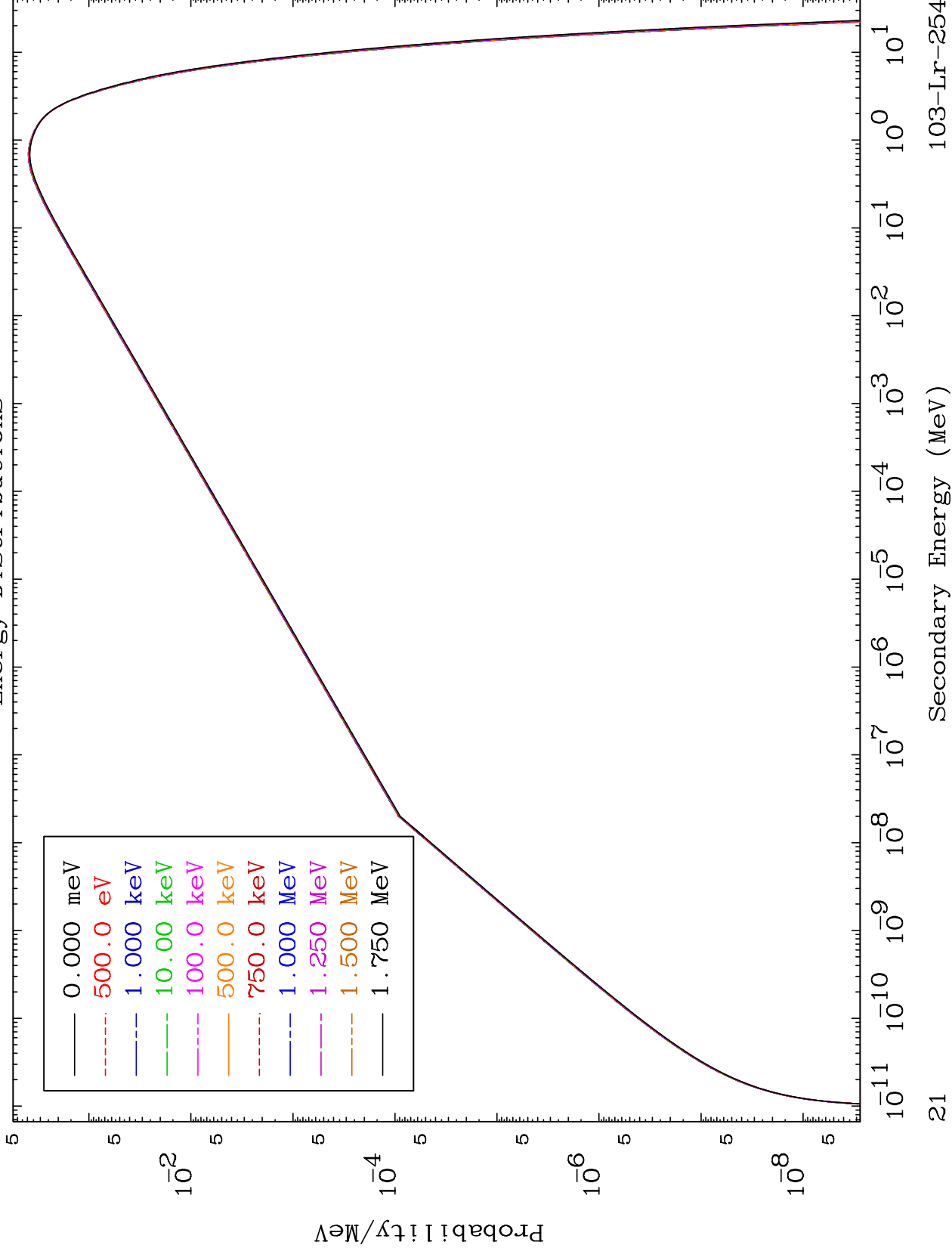
Incident Energy (MeV)

103-Lr-254

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### Fission Energy Distributions

103-Lr-254

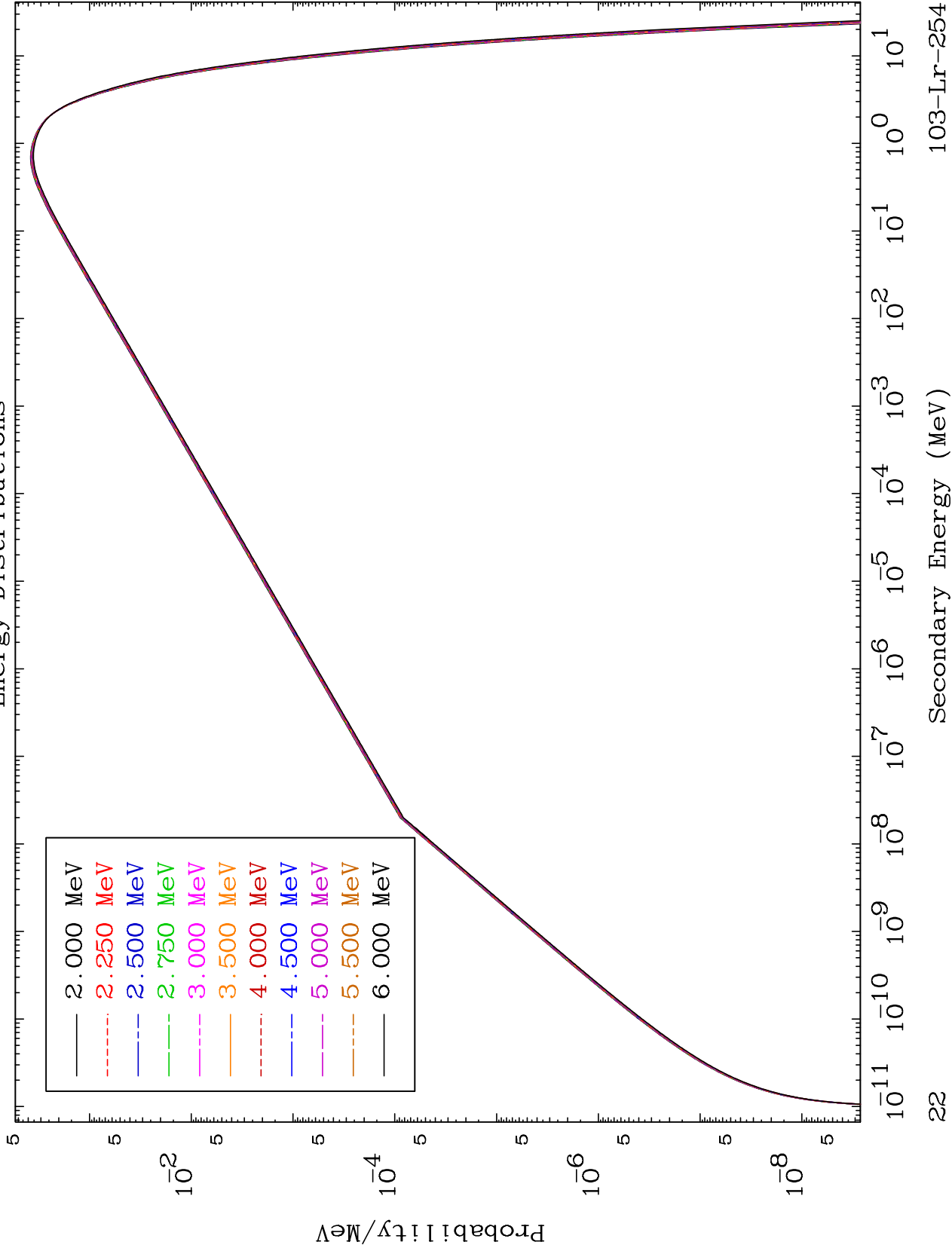


103-Lr-254

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Fission Energy Distributions

103-Lr-254



103-Lr-254

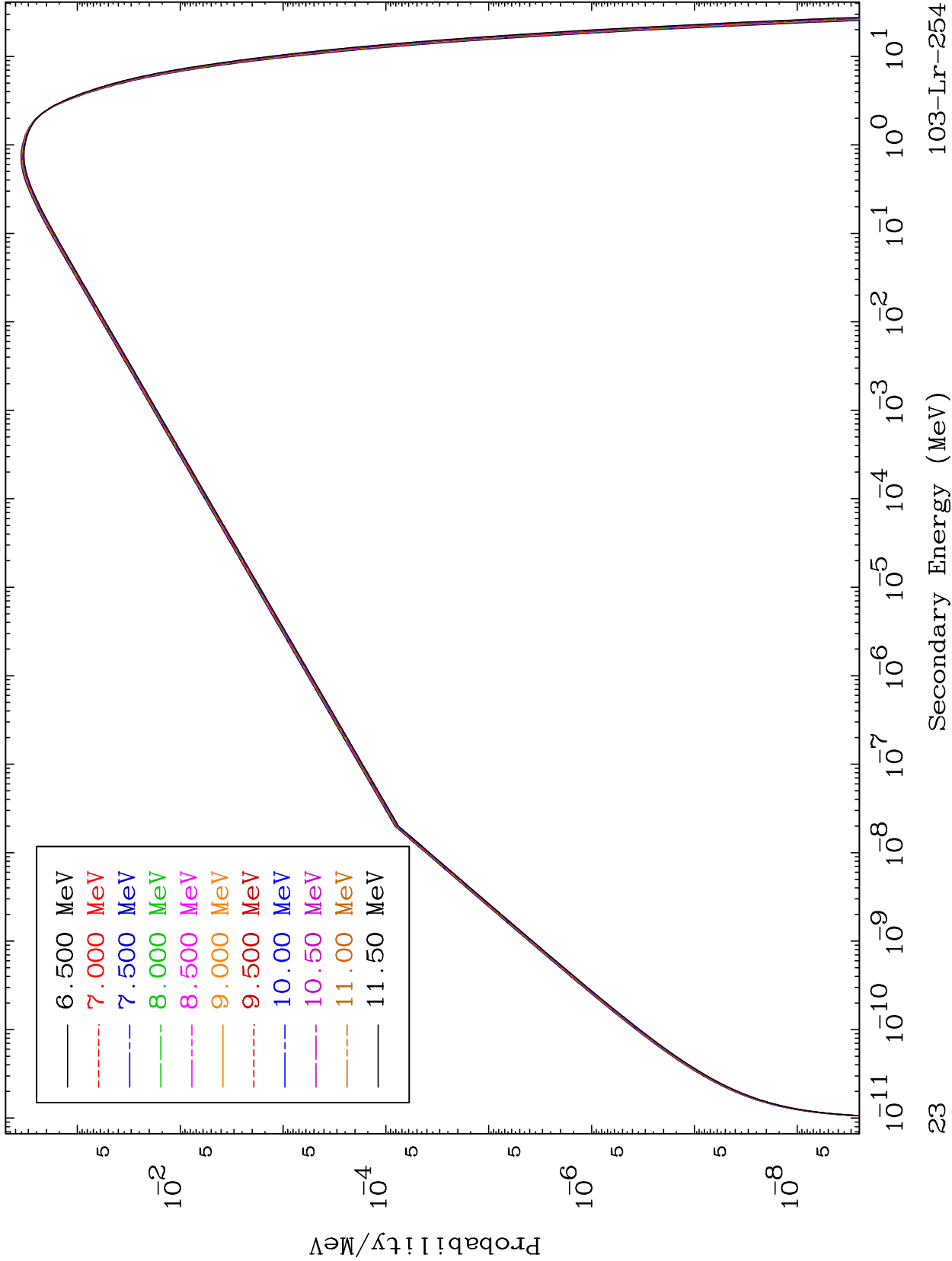
Secondary Energy (MeV)

22

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### Fission Energy Distributions

103-Lr-254

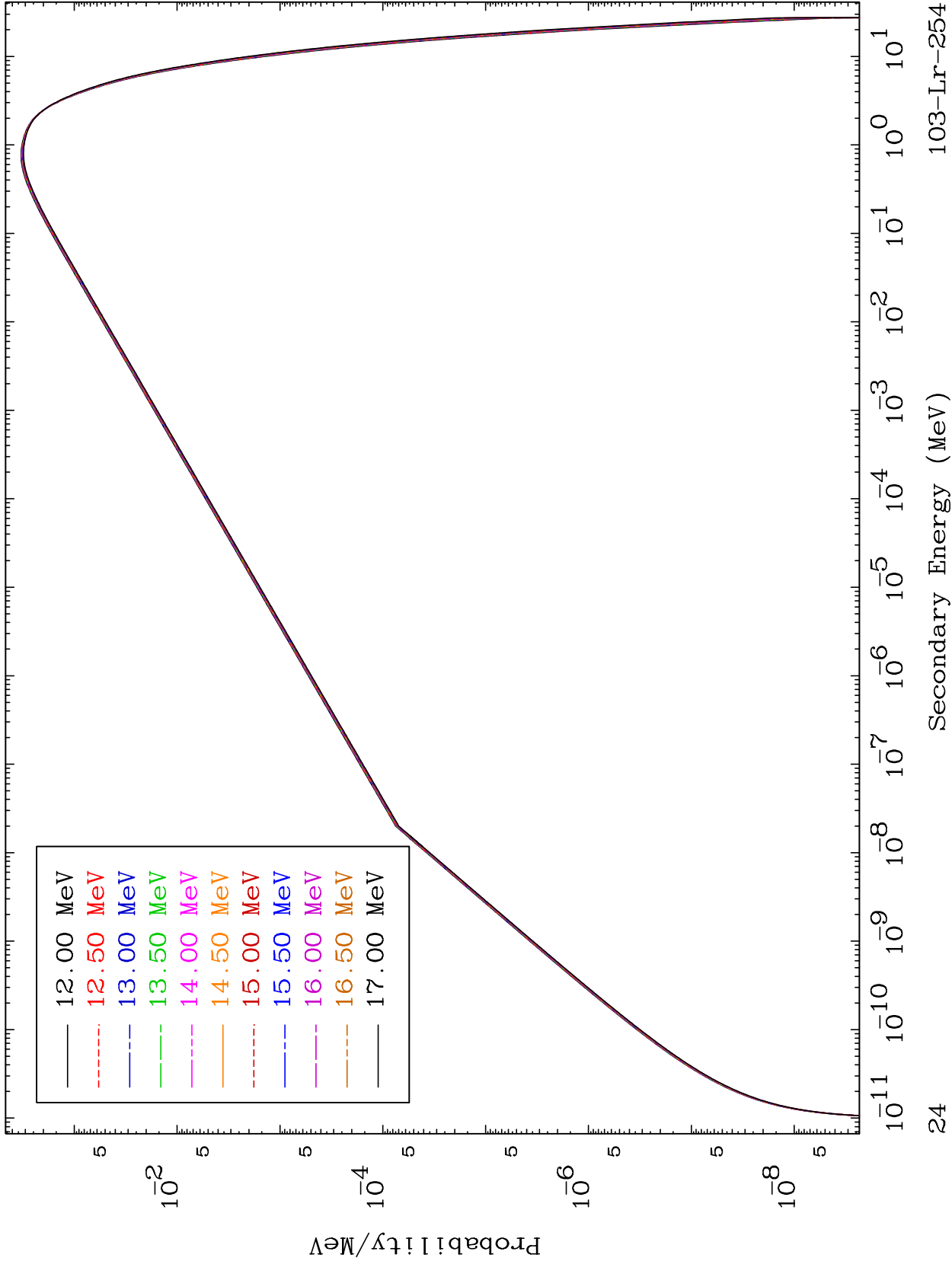


103-Lr-254

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### Fission Energy Distributions

103-Lr-254



103-Lr-254

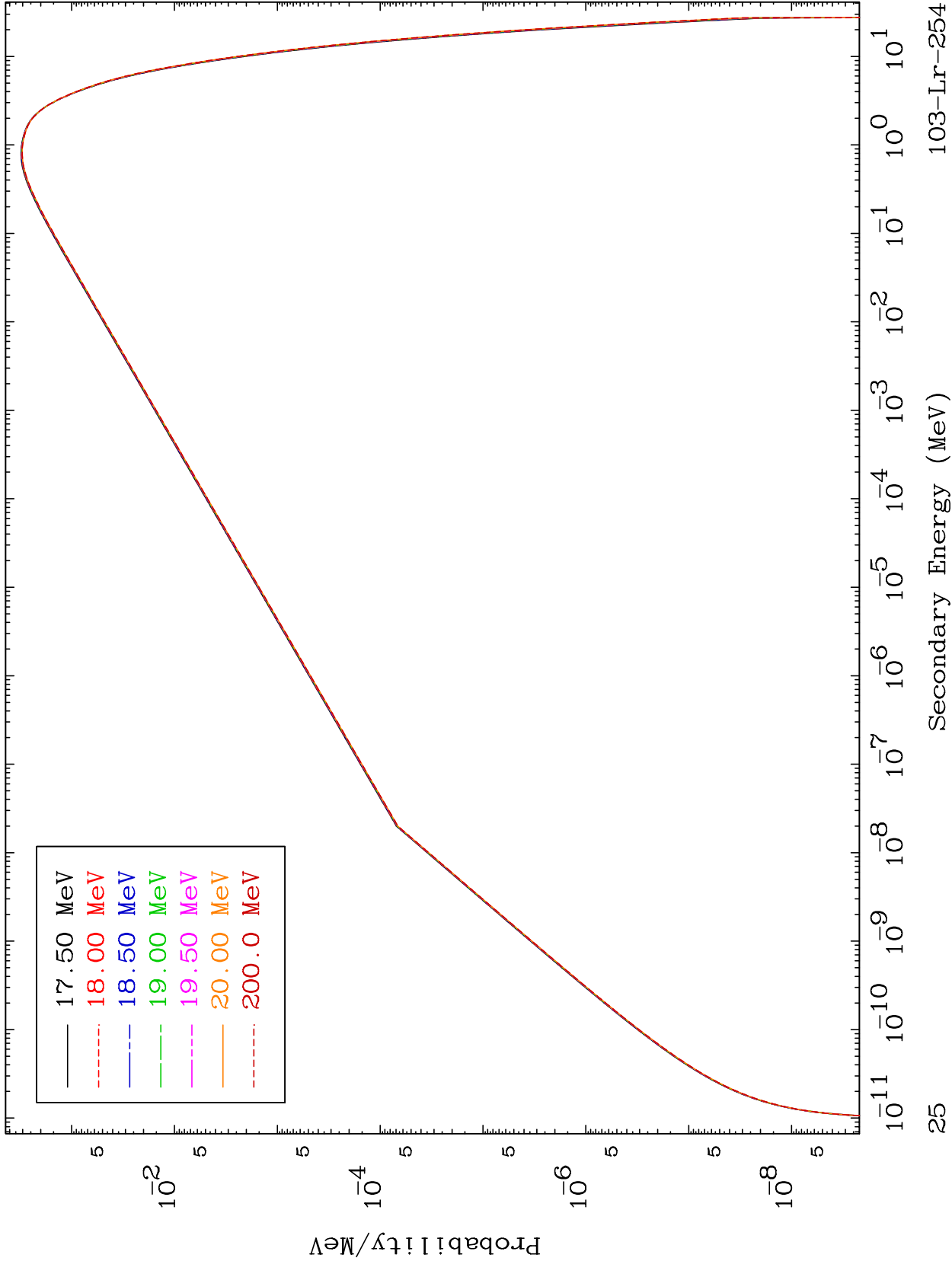
Secondary Energy (MeV)

24

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Fission Energy Distributions

103-Lr-254

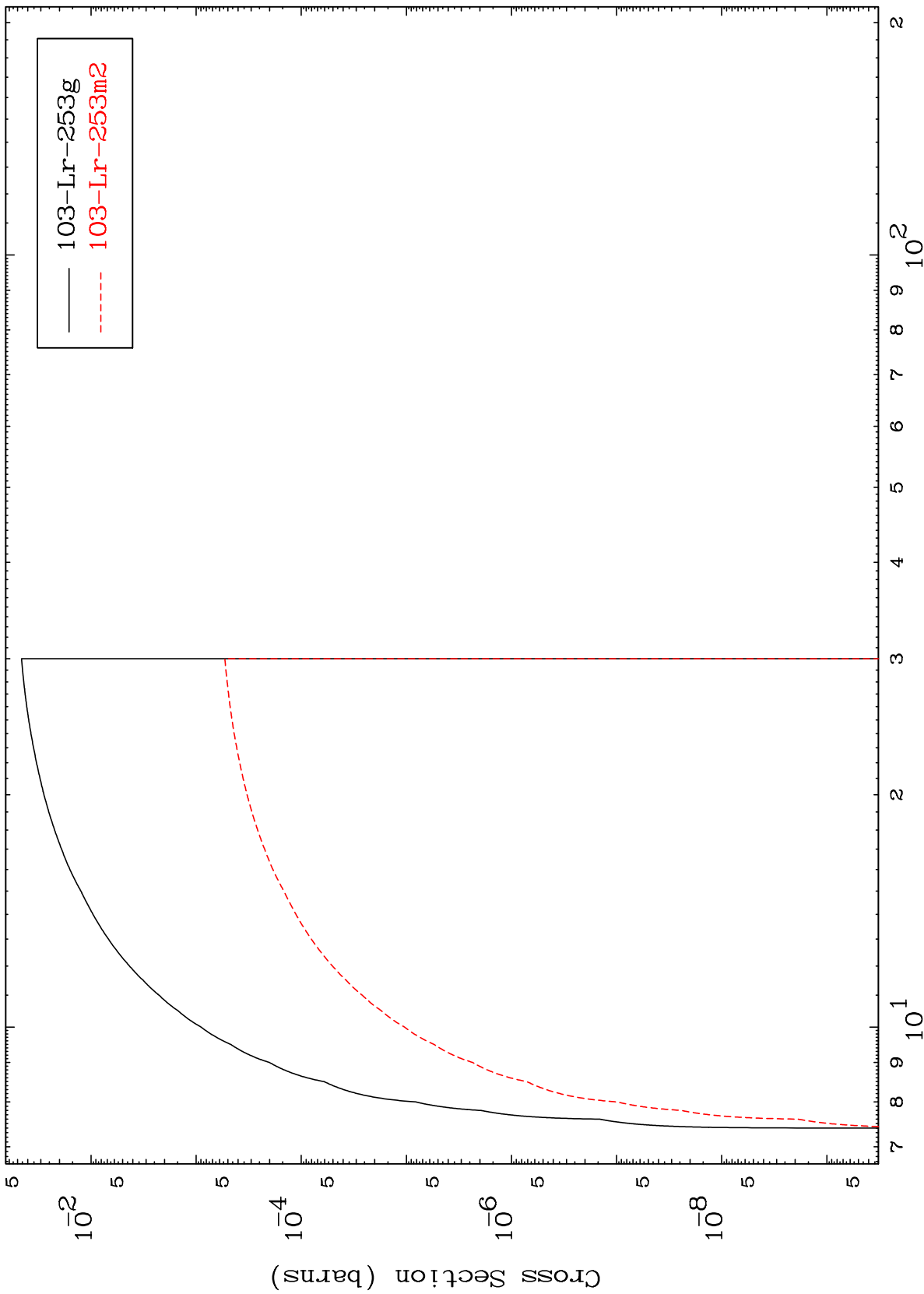


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103-Lr-254

(n,2n)

Radionuclide Production Cross Section



26

Incident Energy (MeV)

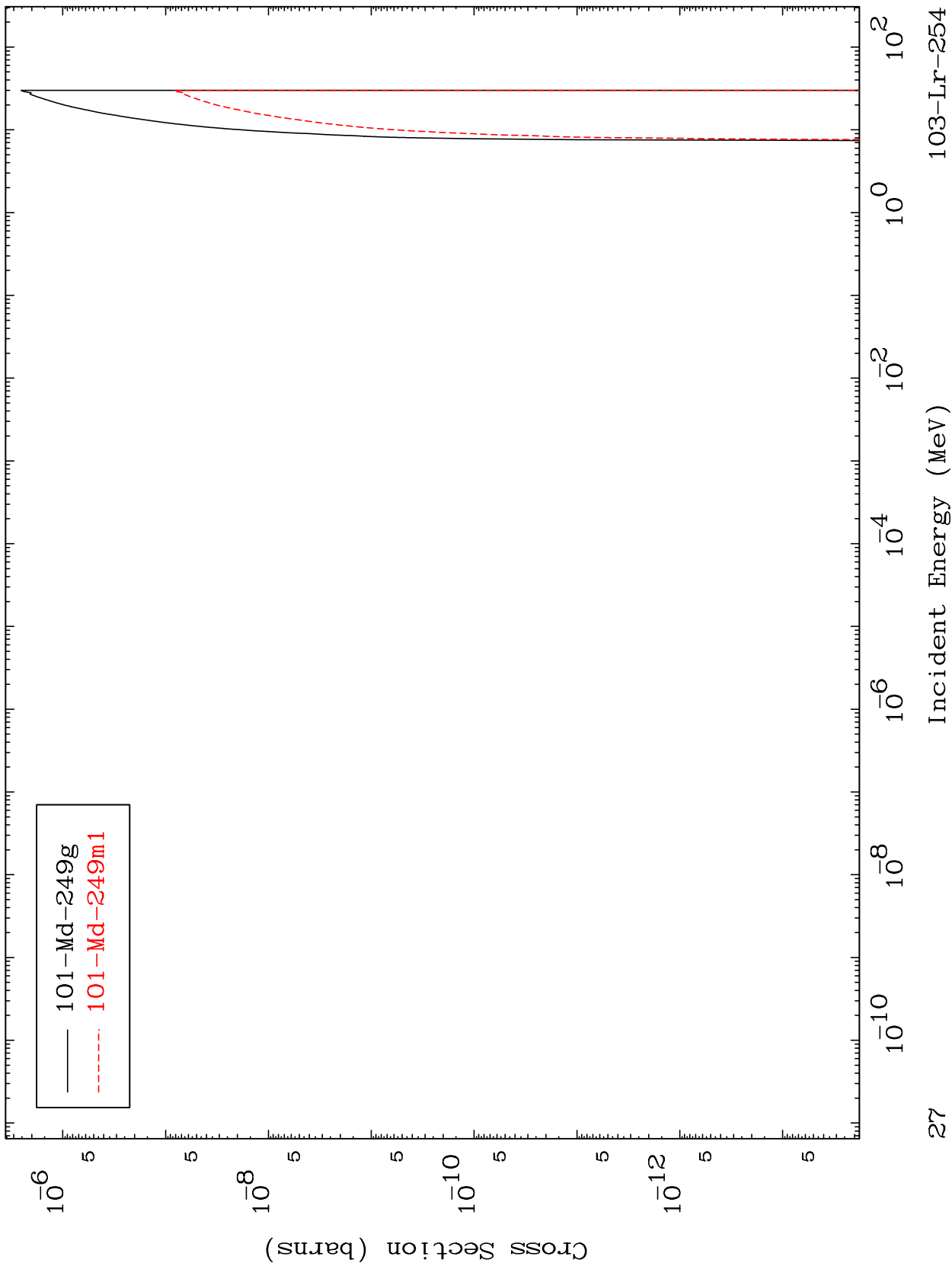
103-Lr-254

MAT 9983

(n,2n)  $\alpha$

103-Lr-254

Radionuclide Production Cross Section

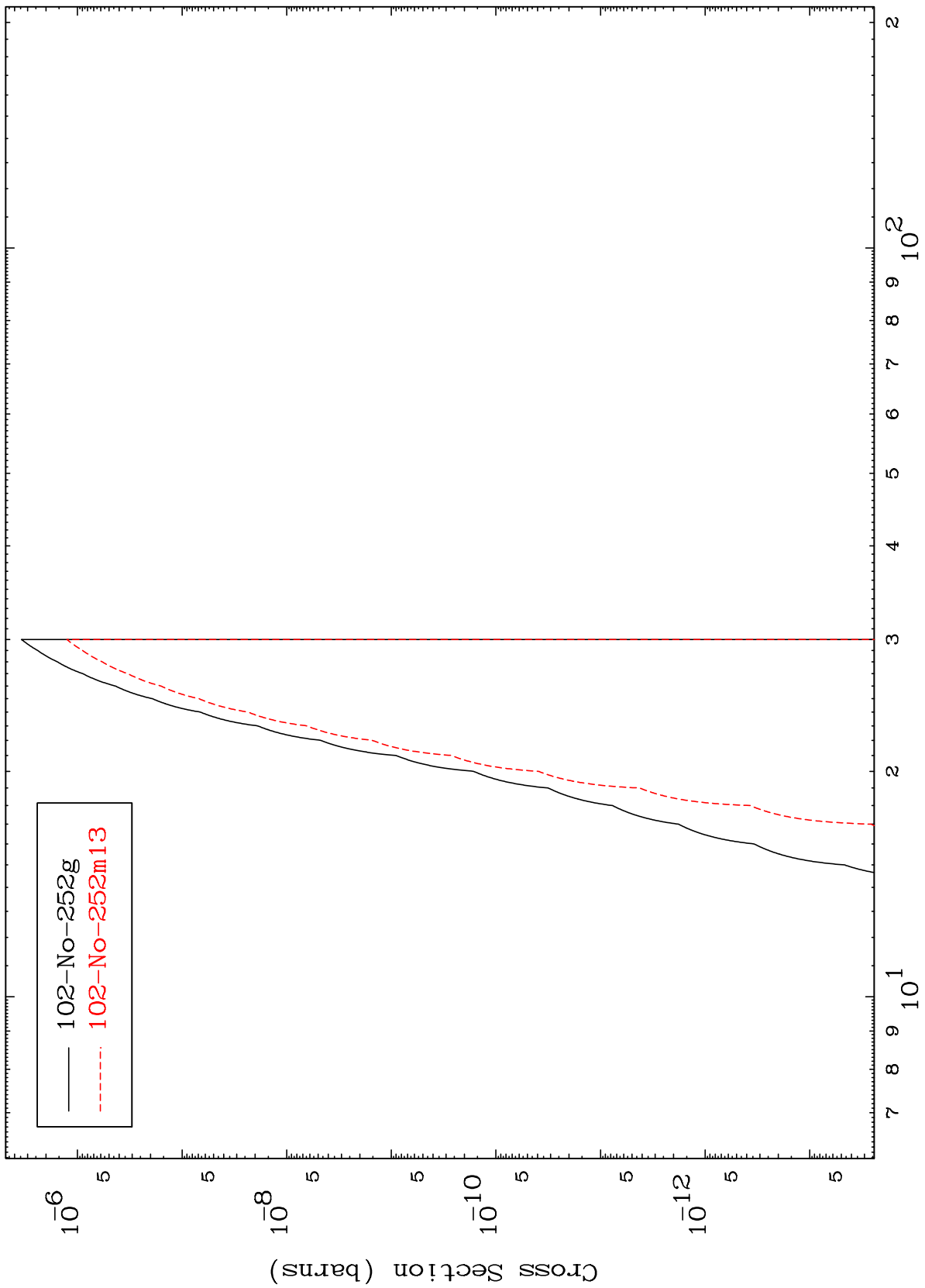


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(n,n') d

103-Lr-254

Radionuclide Production Cross Section



28

Incident Energy (MeV)

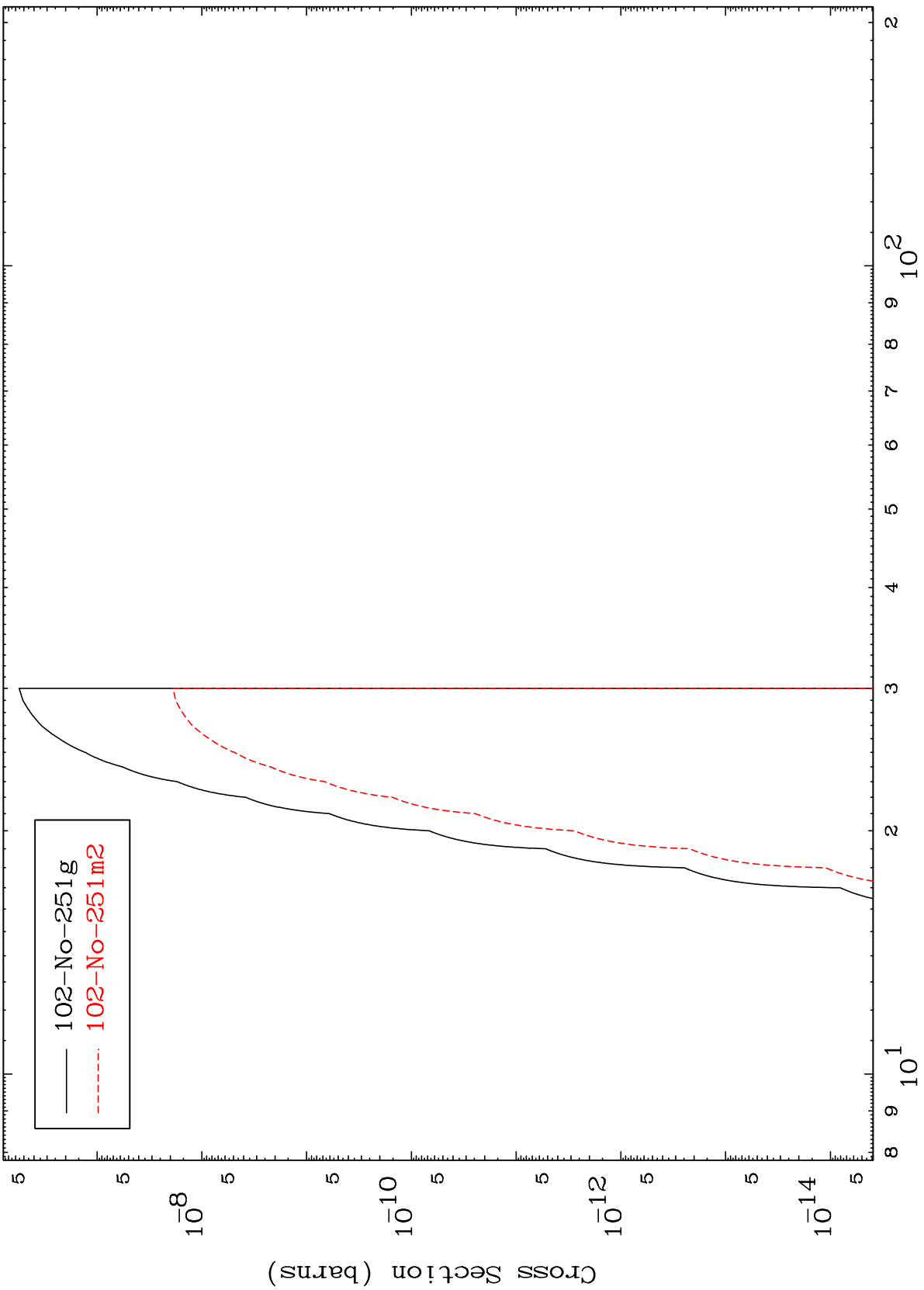
103-Lr-254

MAT 9983

(n,n') t

103-Lr-254

Radionuclide Production Cross Section



29

Incident Energy (MeV)

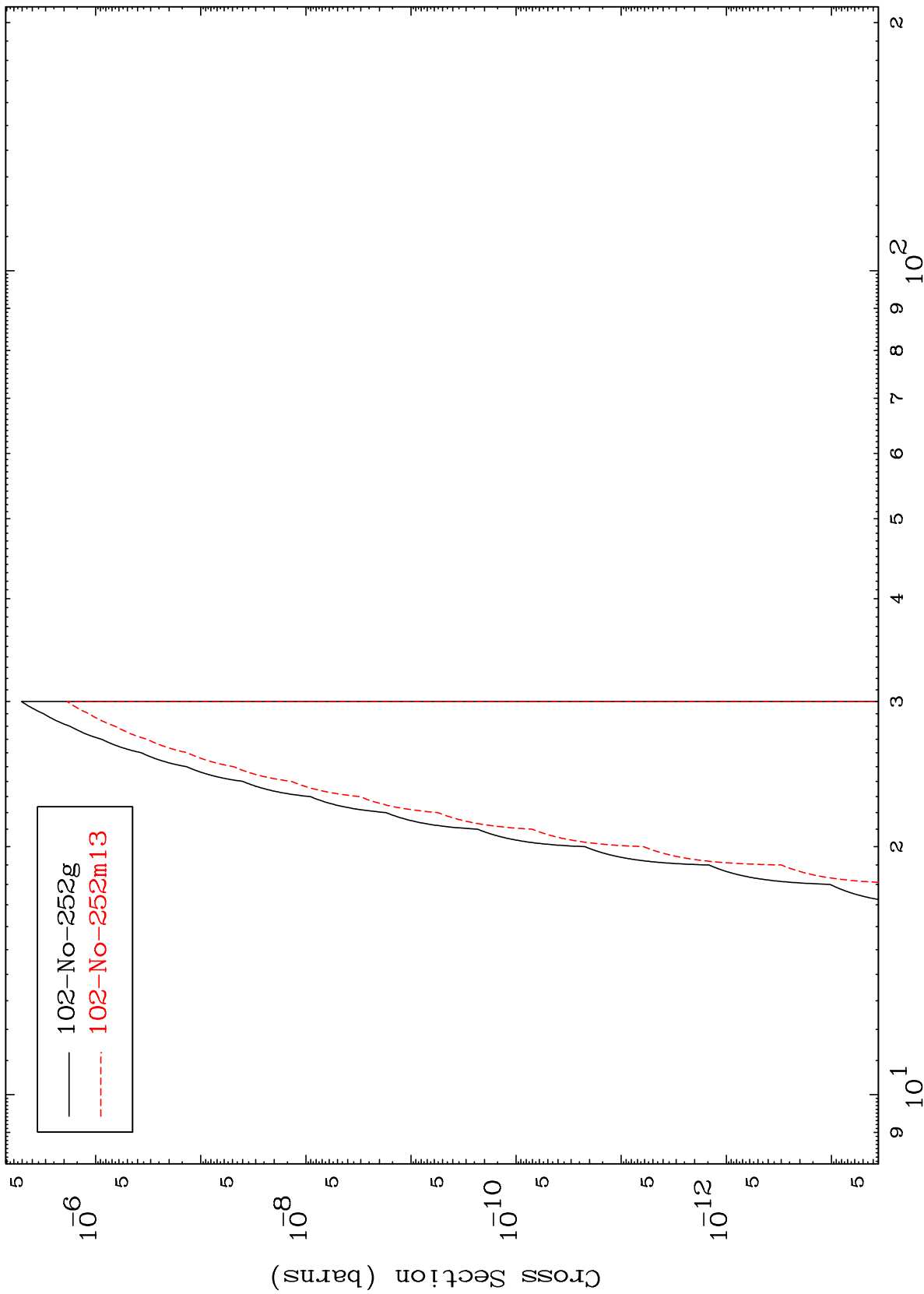
103-Lr-254

MAT 9983

(n,2n) p

103-Lr-254

Radionuclide Production Cross Section



Incident Energy (MeV)

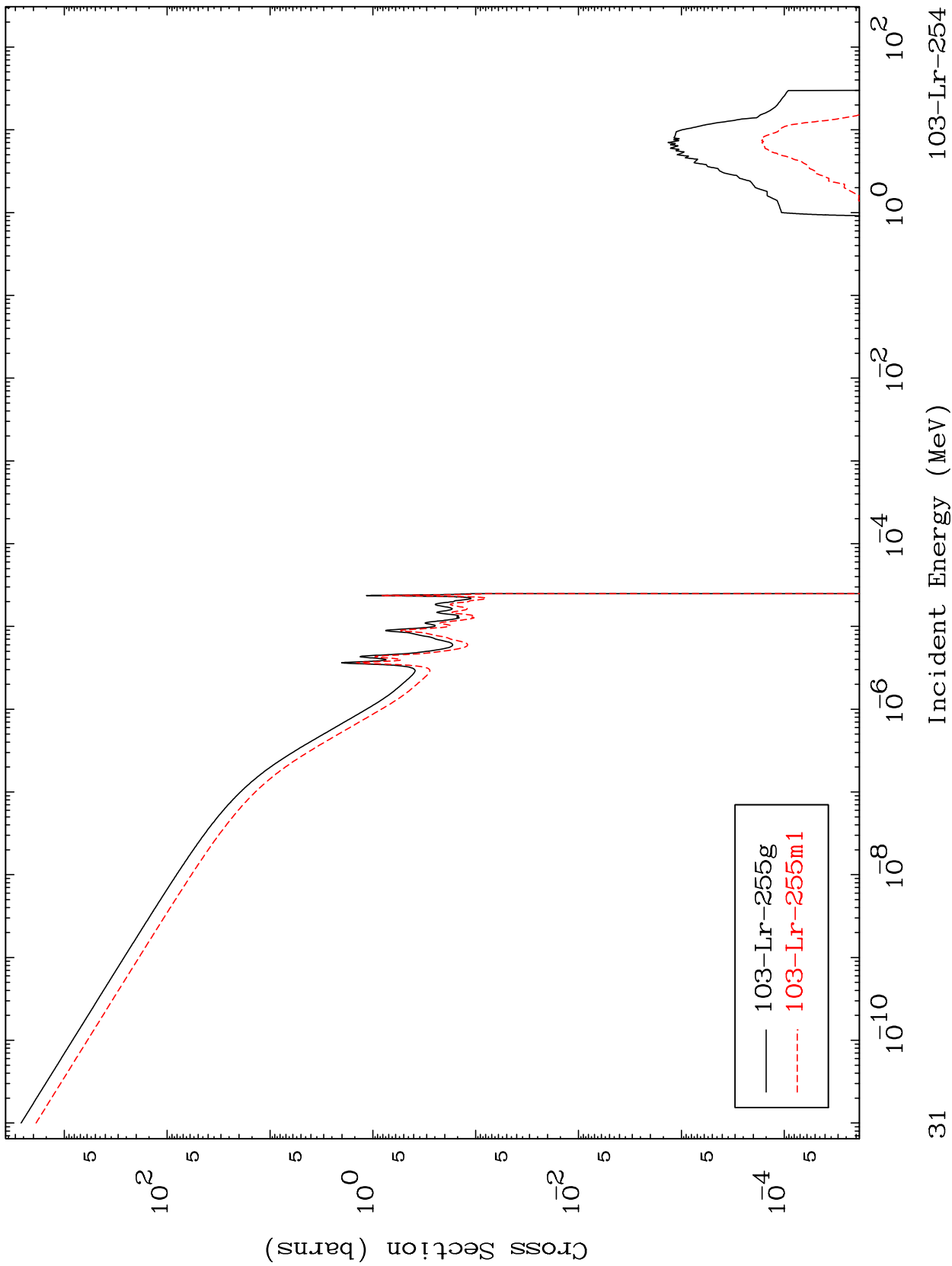
103-Lr-254

30

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103-Lr-254

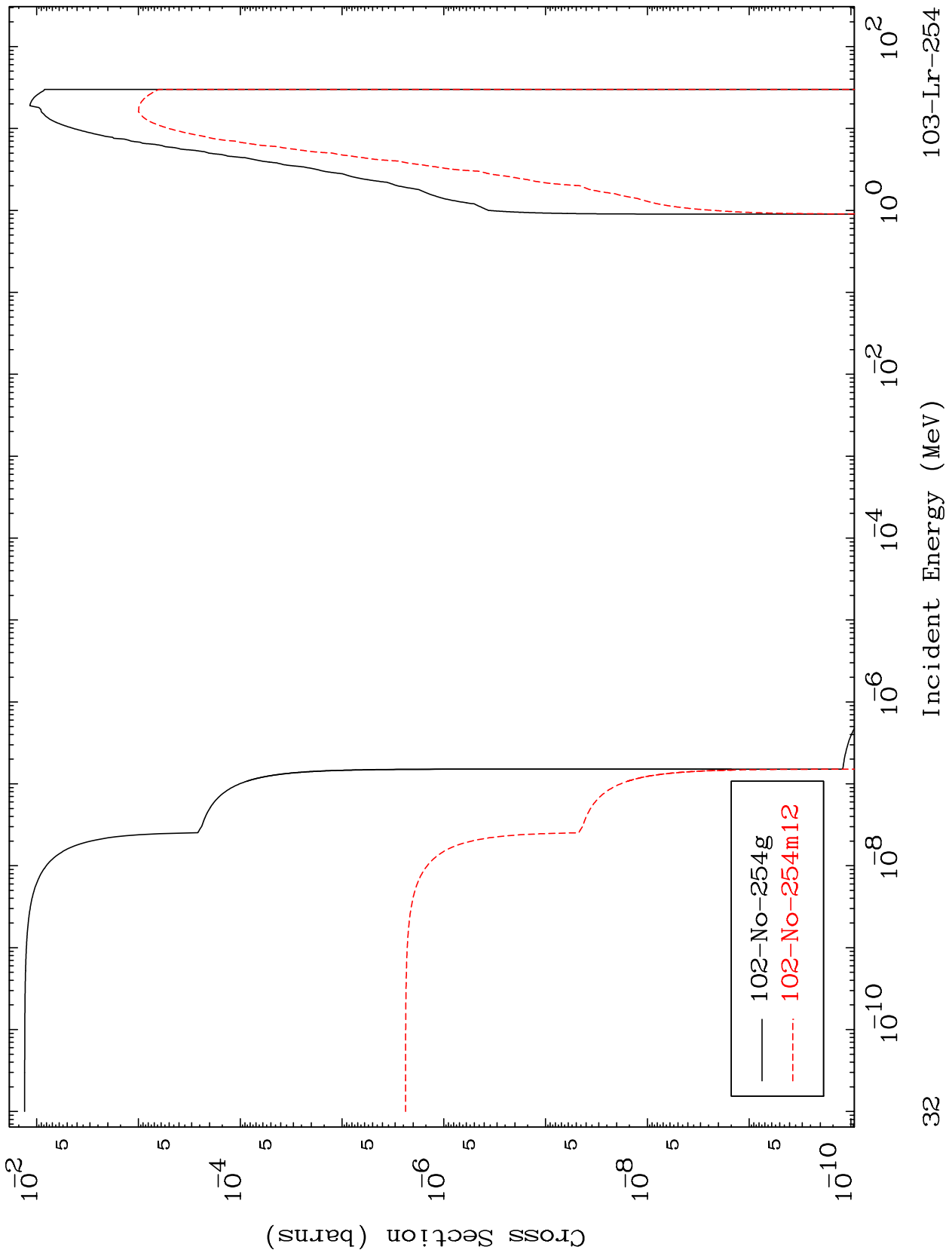
(n,  $\gamma$ )  
Radionuclide Production Cross Section



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103-Lr-254

(n,p)  
Radionuclide Production Cross Section

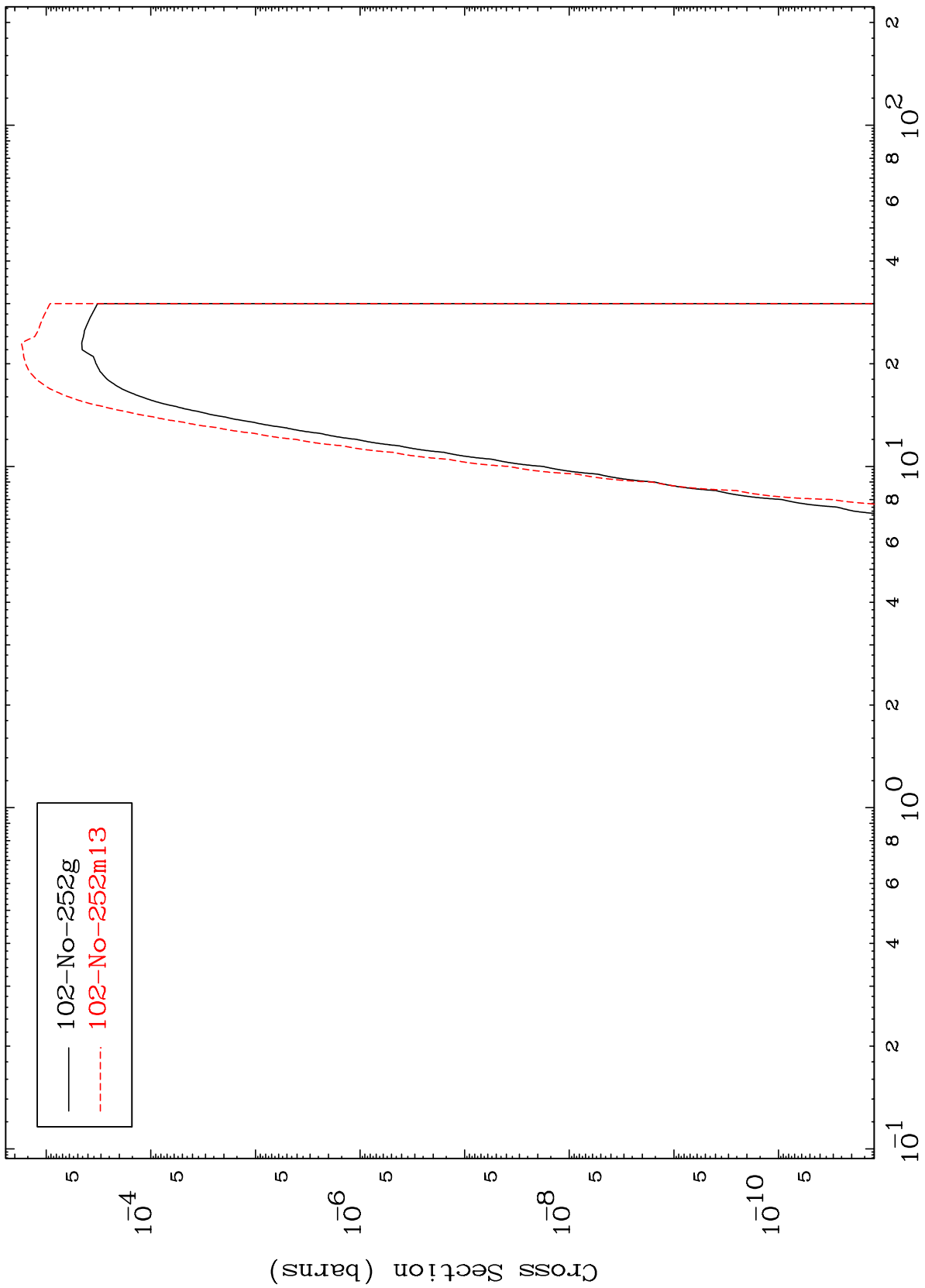
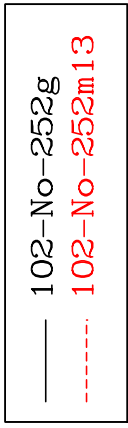


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(n, t)

103-Lr-254

Radionuclide Production Cross Section



Incident Energy (MeV)

103-Lr-254

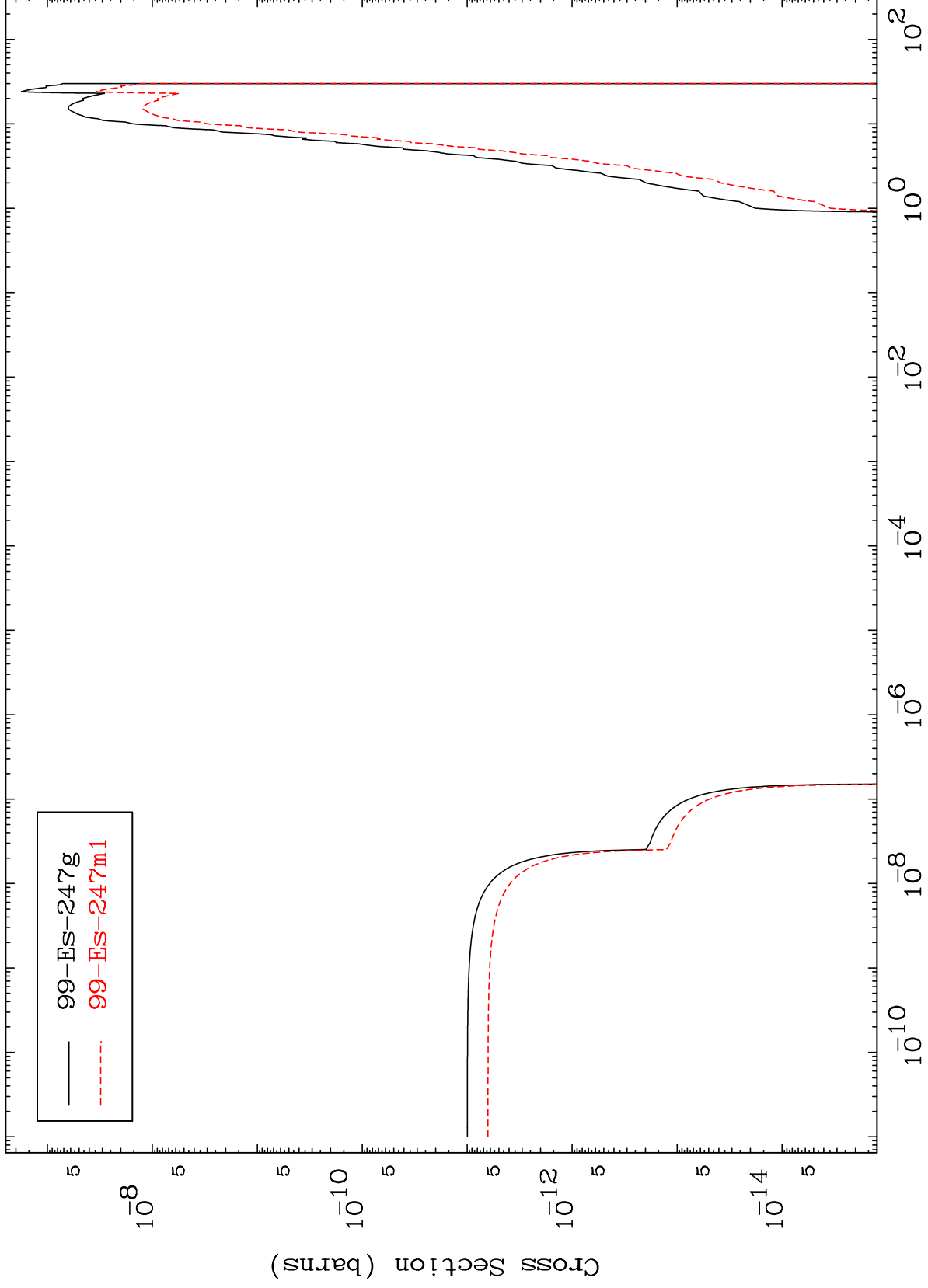
33

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(n,2α)

103-Lr-254

Radionuclide Production Cross Section



34

Incident Energy (MeV)

103-Lr-254