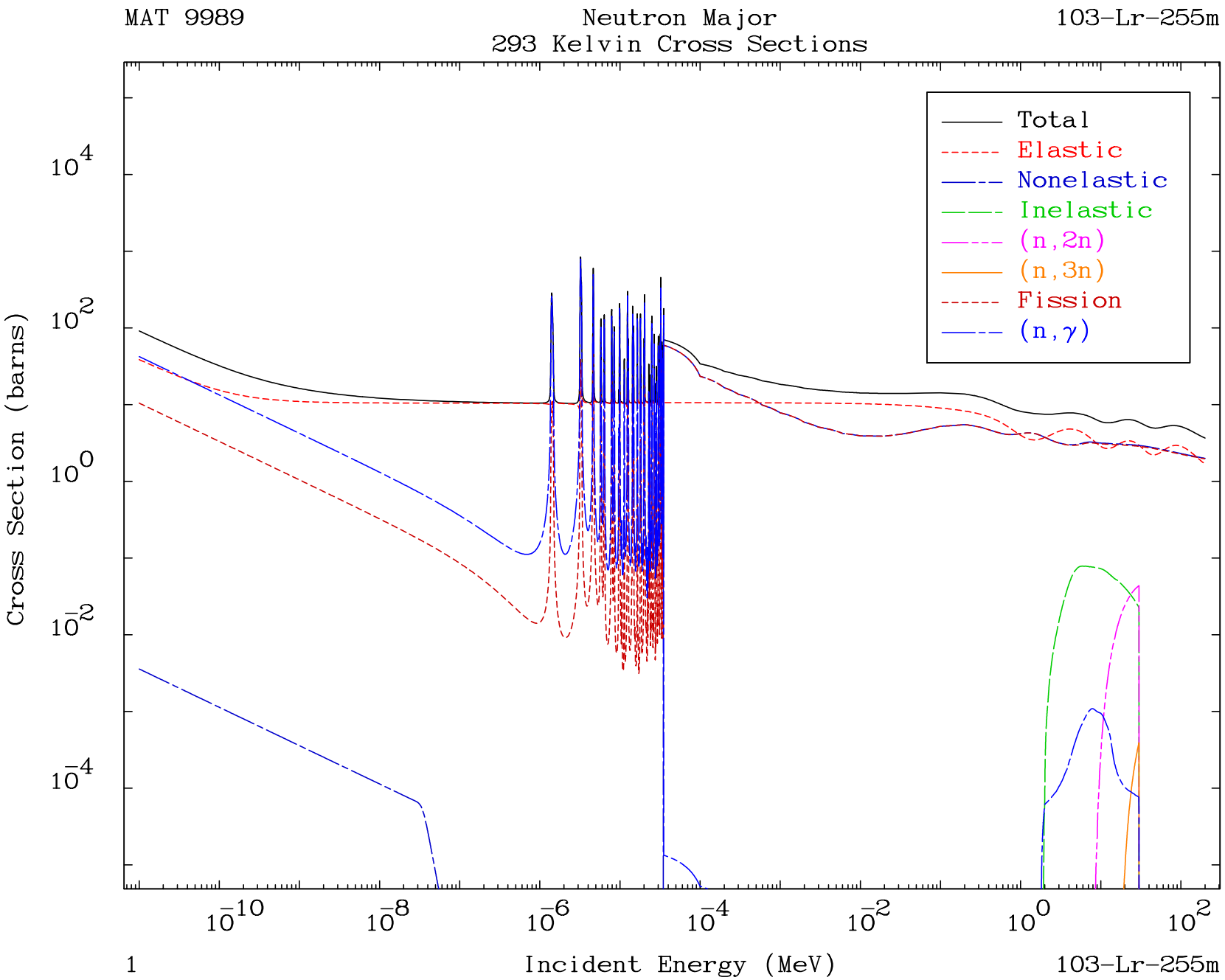
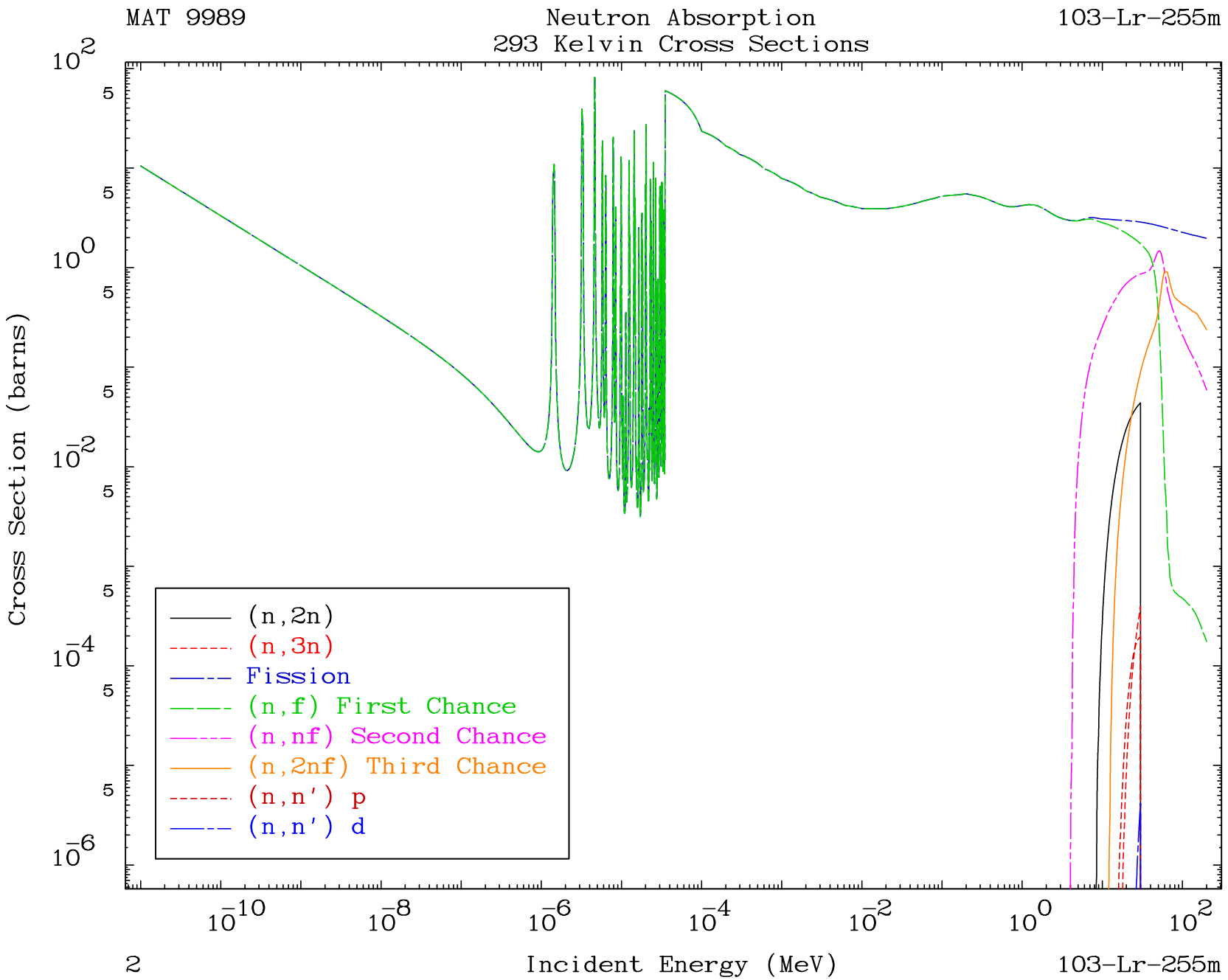
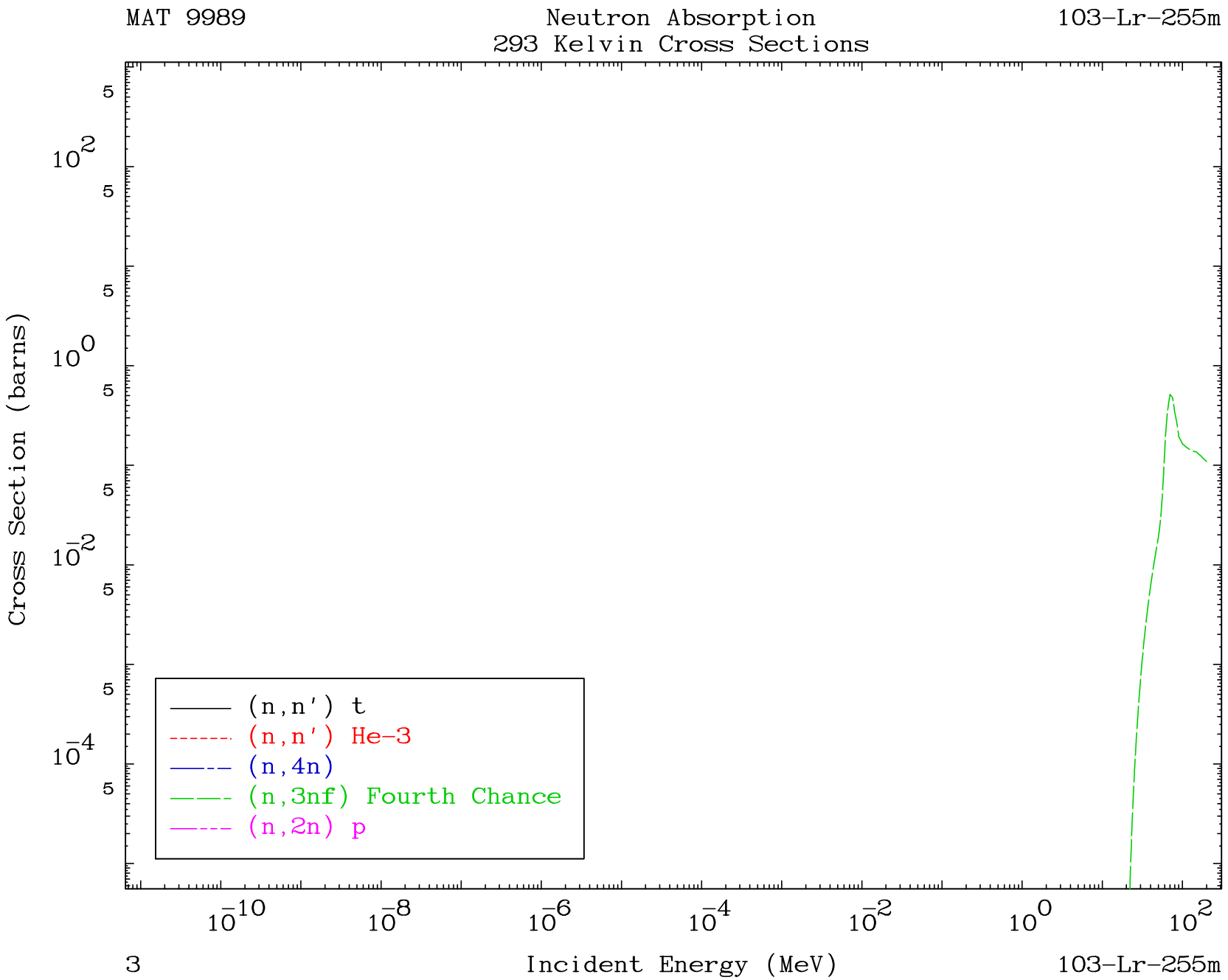
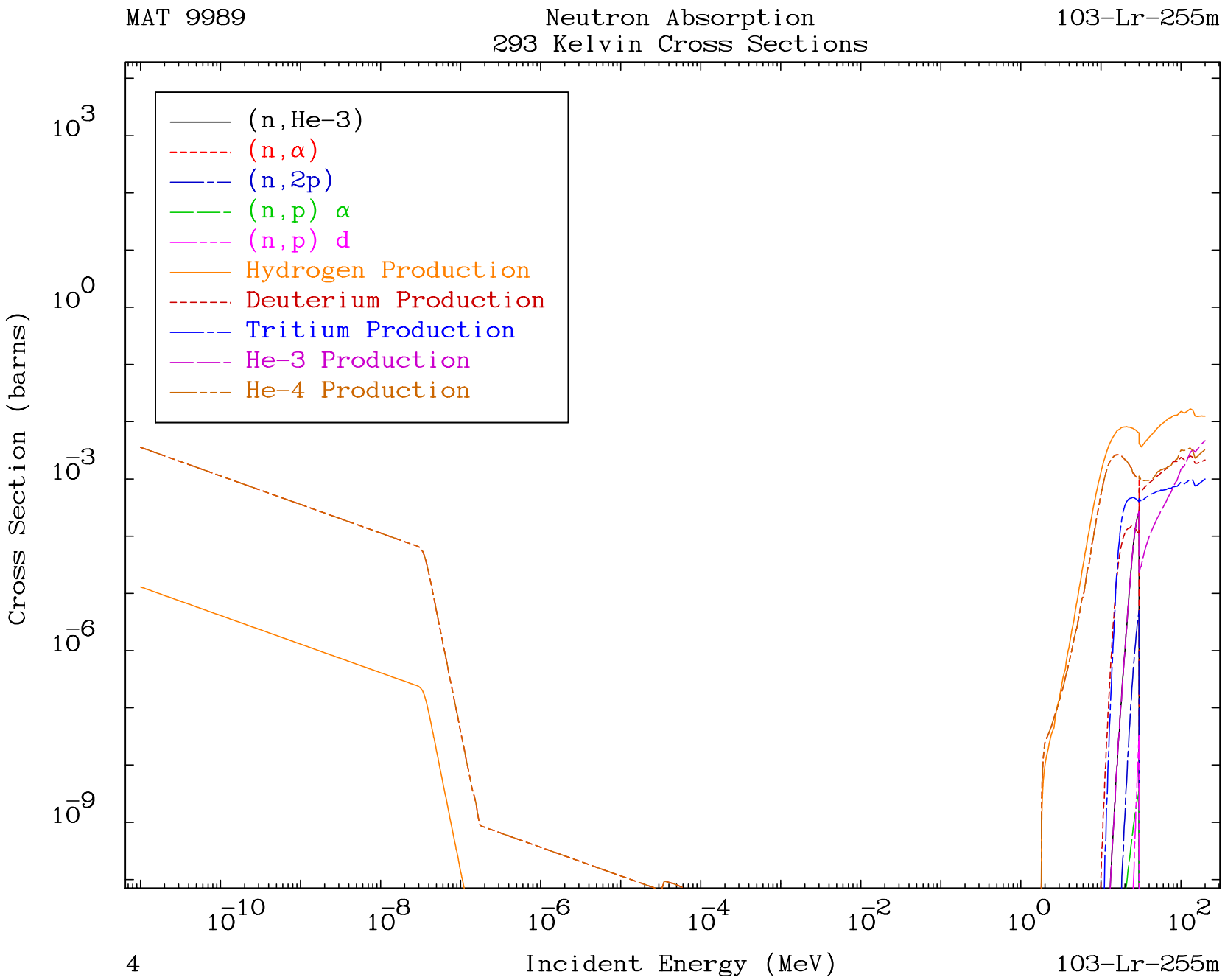
Program EVALPLOT
(Version 2021-1)
by
Dermott E. Cullen
(Present Contact Information)
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.
Tele: 925-443-1911
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

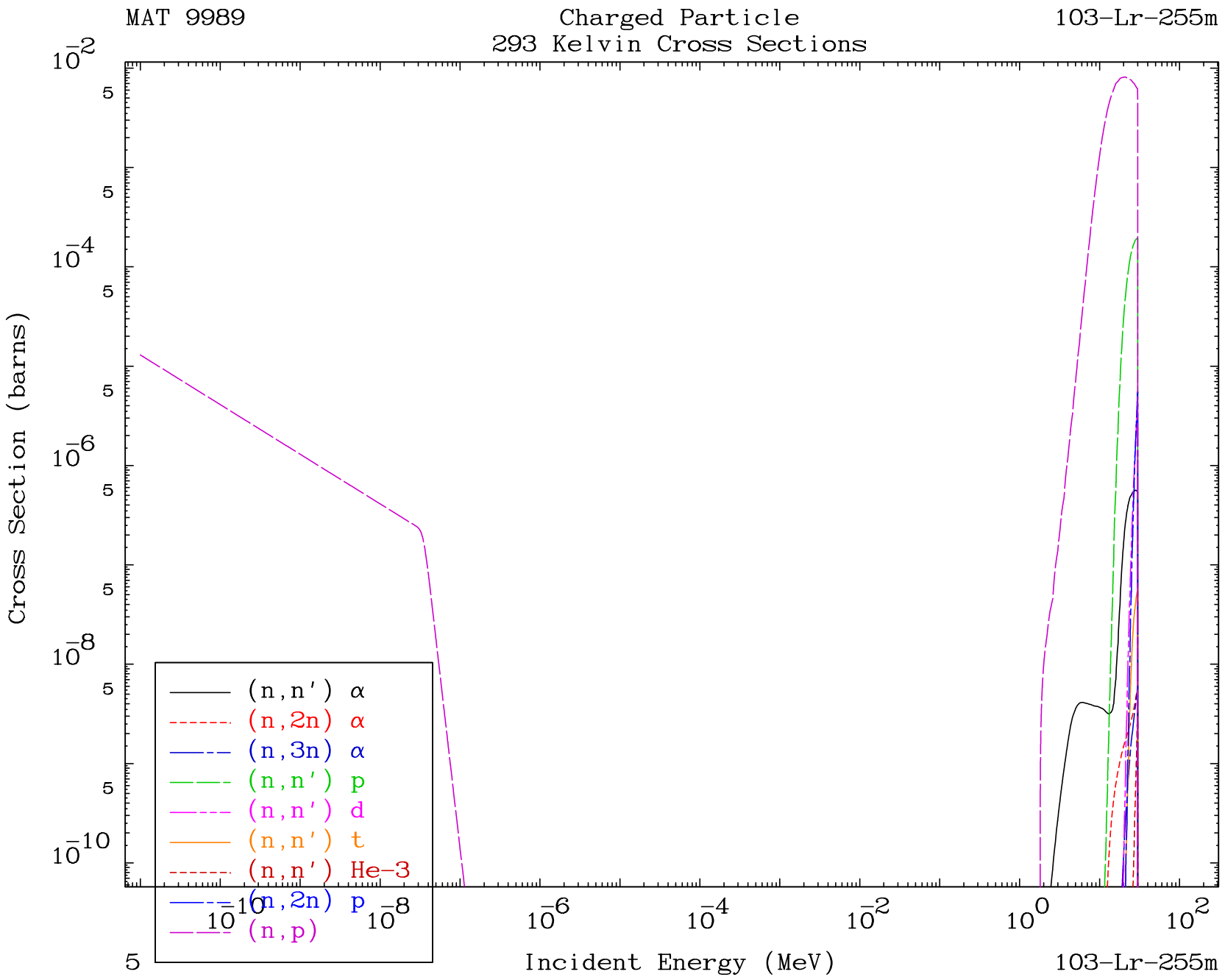
Press Mouse Button to Start

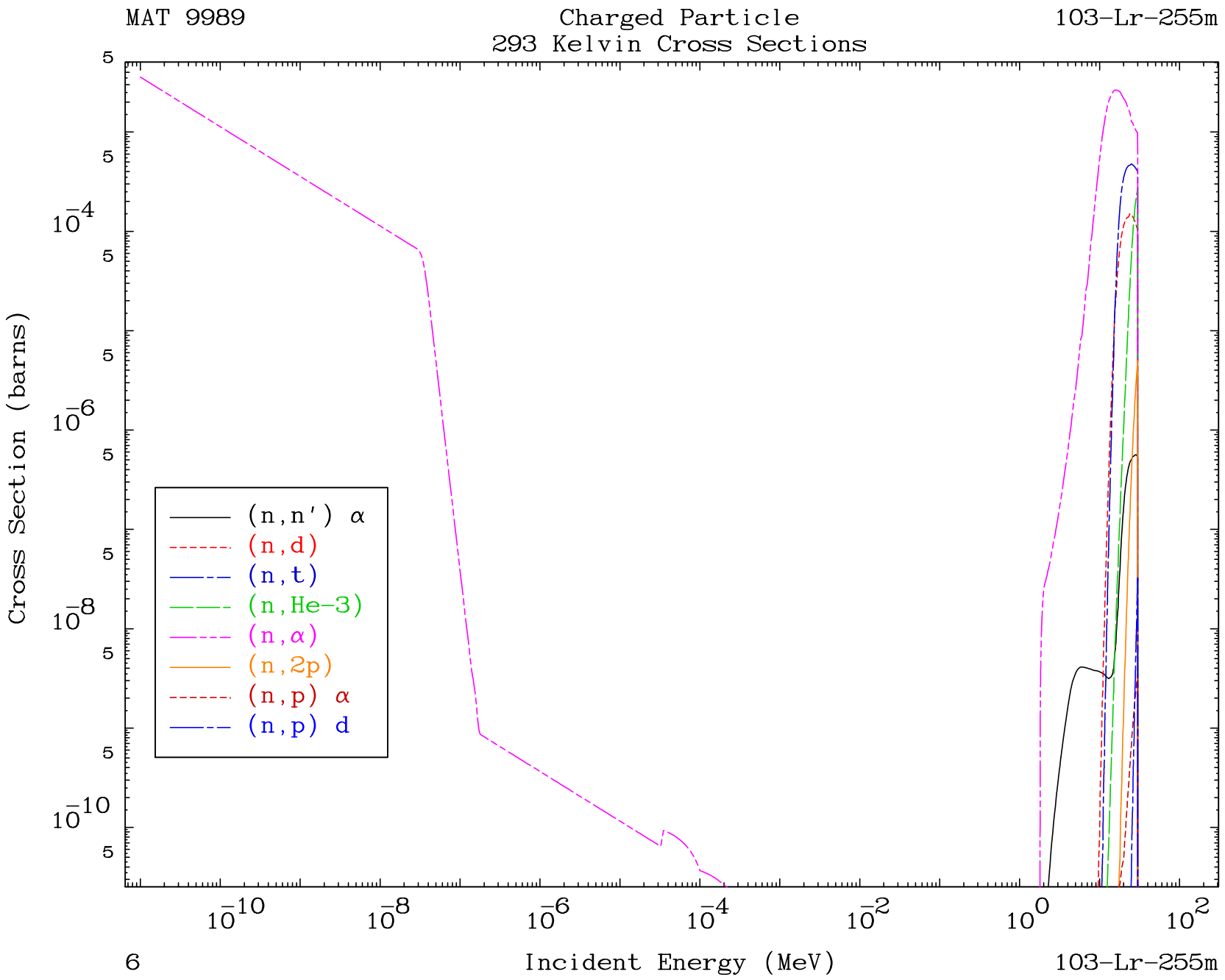

MAT 9989
Neutron Major
293 Kelvin Cross Sections
103-Lr-255m
Total
Elastic
Nonelastic
Inelastic
(n,2n)
(n,3n)
Fission
(n,γ)
Cross Section (barns)
10^4
10^2
10^0
10^-2
10^-4
10^-10
10^-8
10^-6
10^-4
10^-2
10^0
10^2
Incident Energy (MeV)
1
103-Lr-255m

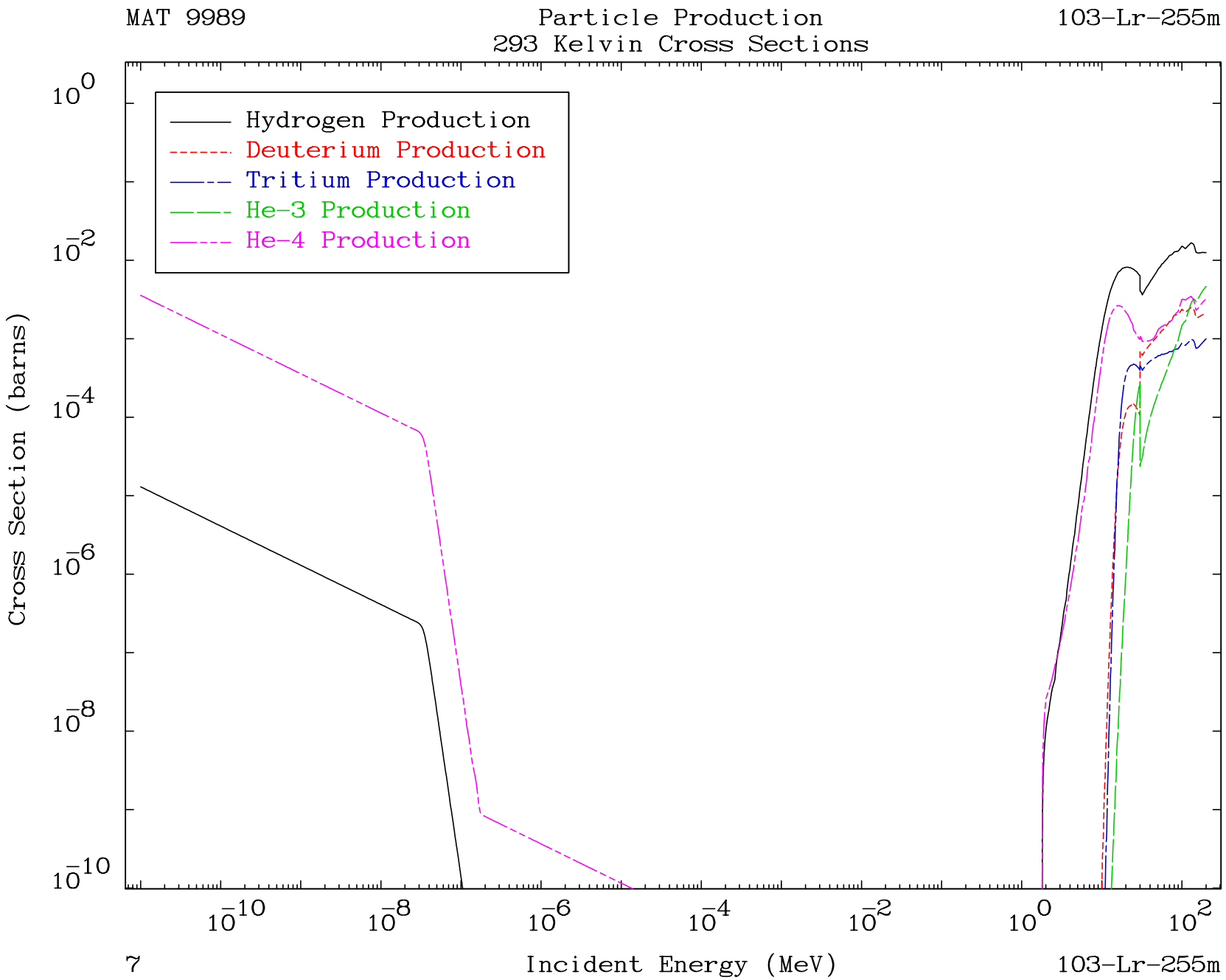

MAT 9989
Neutron Absorption
293 Kelvin Cross Sections
103-Lr-255m
Cross Section (barns)
Incident Energy (MeV)
(n,2n)
(n,3n)
Fission
(n,f) First Chance
(n,nf) Second Chance
(n,2nf) Third Chance
(n,n') p
(n,n') d
2
103-Lr-255m

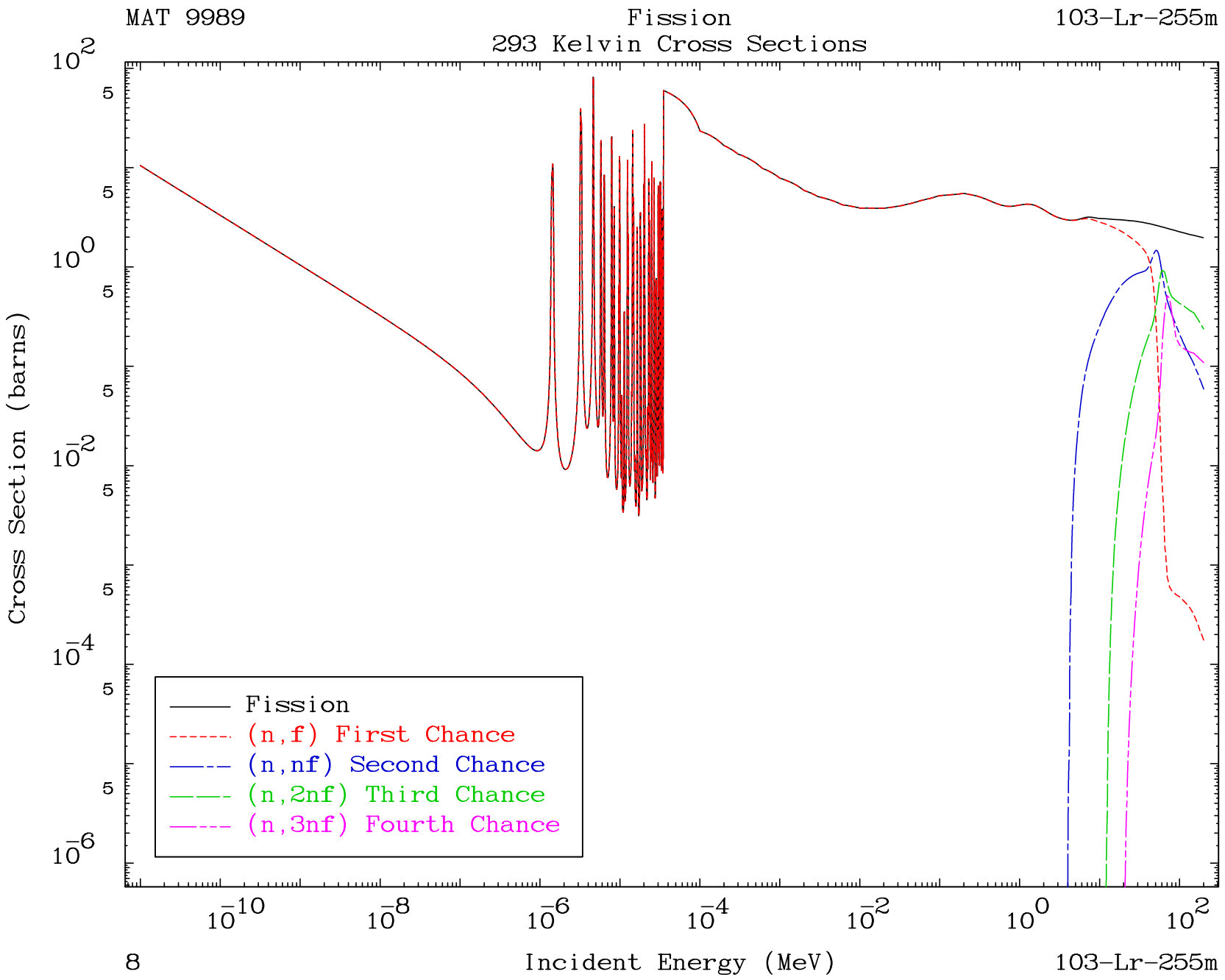

MAT 9989
Neutron Absorption
293 Kelvin Cross Sections
103-Lr-255m
Cross Section (barns)
10^2
10^0
10^-2
10^-4
5
(n,n') t
(n,n') He-3
(n,4n)
(n,3nf) Fourth Chance
(n,2n) p
10^-10
10^-8
10^-6
10^-4
10^-2
10^0
10^2
Incident Energy (MeV)
3
103-Lr-255m

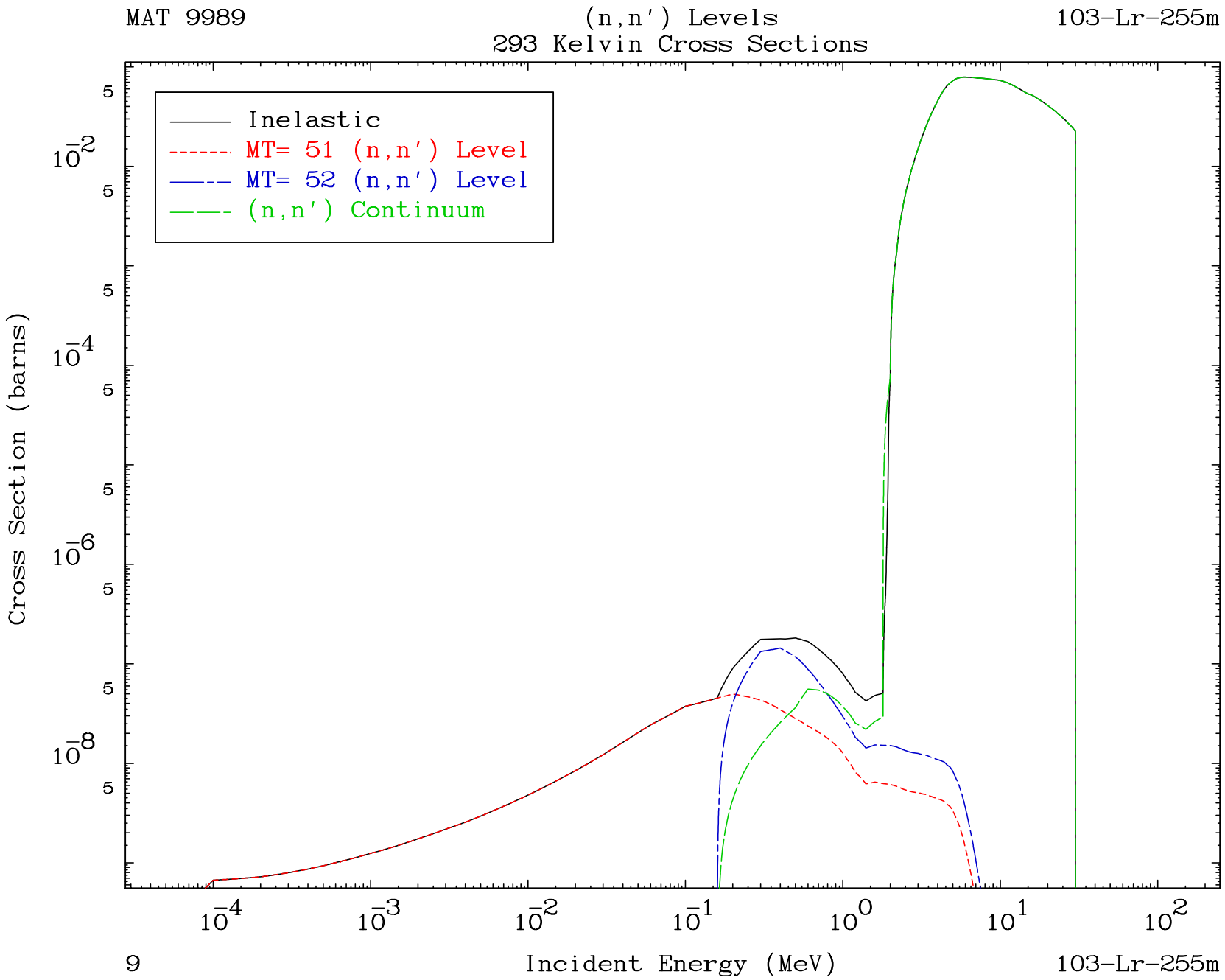

MAT 9989
Neutron Absorption
293 Kelvin Cross Sections
103-Lr-255m
(n,He-3)
(n,α)
(n,2p)
(n,p) α
(n,p) d
Hydrogen Production
Deuterium Production
Tritium Production
He-3 Production
He-4 Production
Cross Section (barns)
10^3
10^0
10^-3
10^-6
10^-9
10^-10
10^-8
10^-6
10^-4
10^-2
10^0
10^2
Incident Energy (MeV)
4
103-Lr-255m

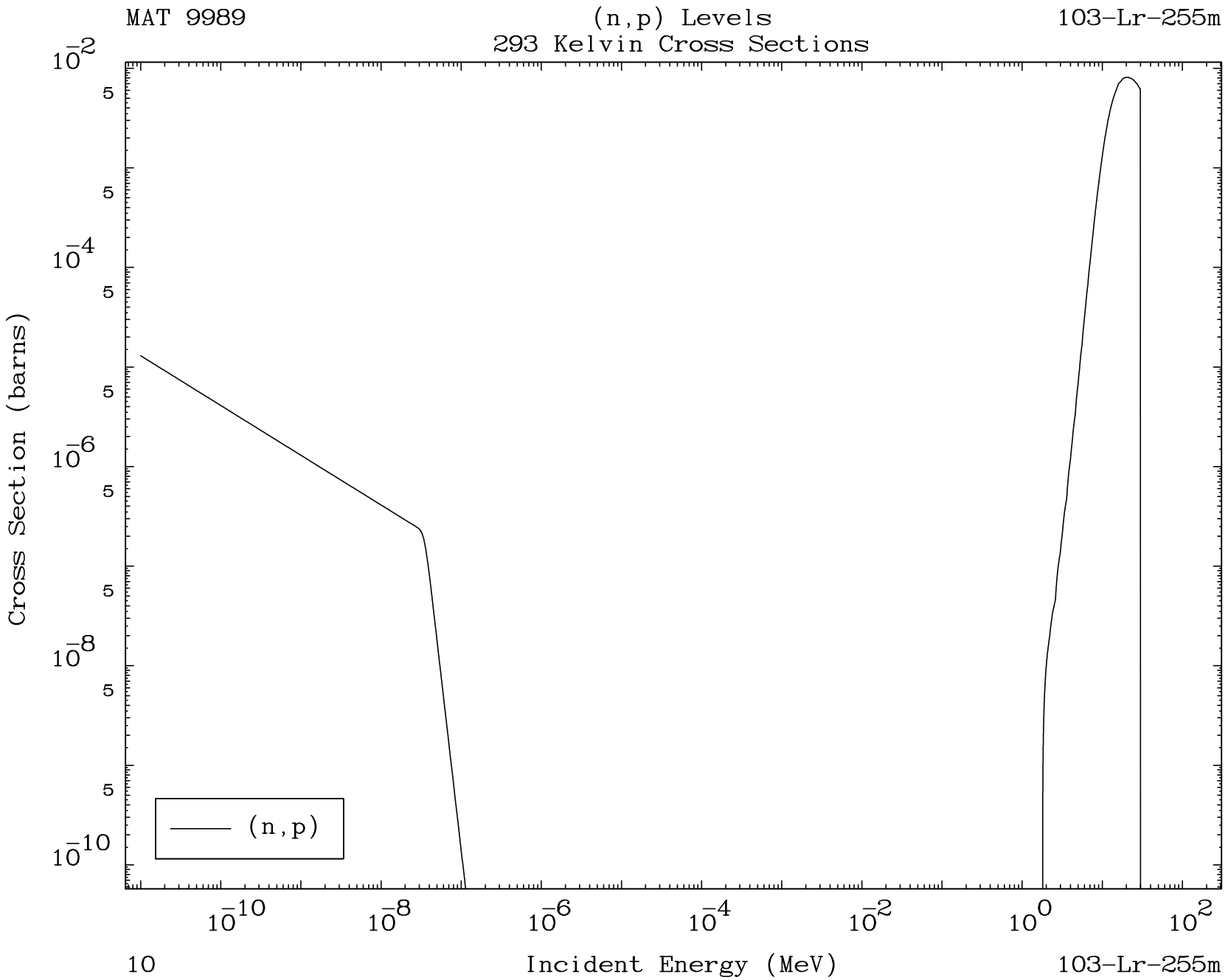

MAT 9989
Charged Particle
293 Kelvin Cross Sections
103-Lr-255m
Cross Section (barns)
Incident Energy (MeV)
103-Lr-255m
(n,n') α
(n,2n) α
(n,3n) α
(n,n') p
(n,n') d
(n,n') t
(n,n') He-3
(n,2n) p
(n,p)

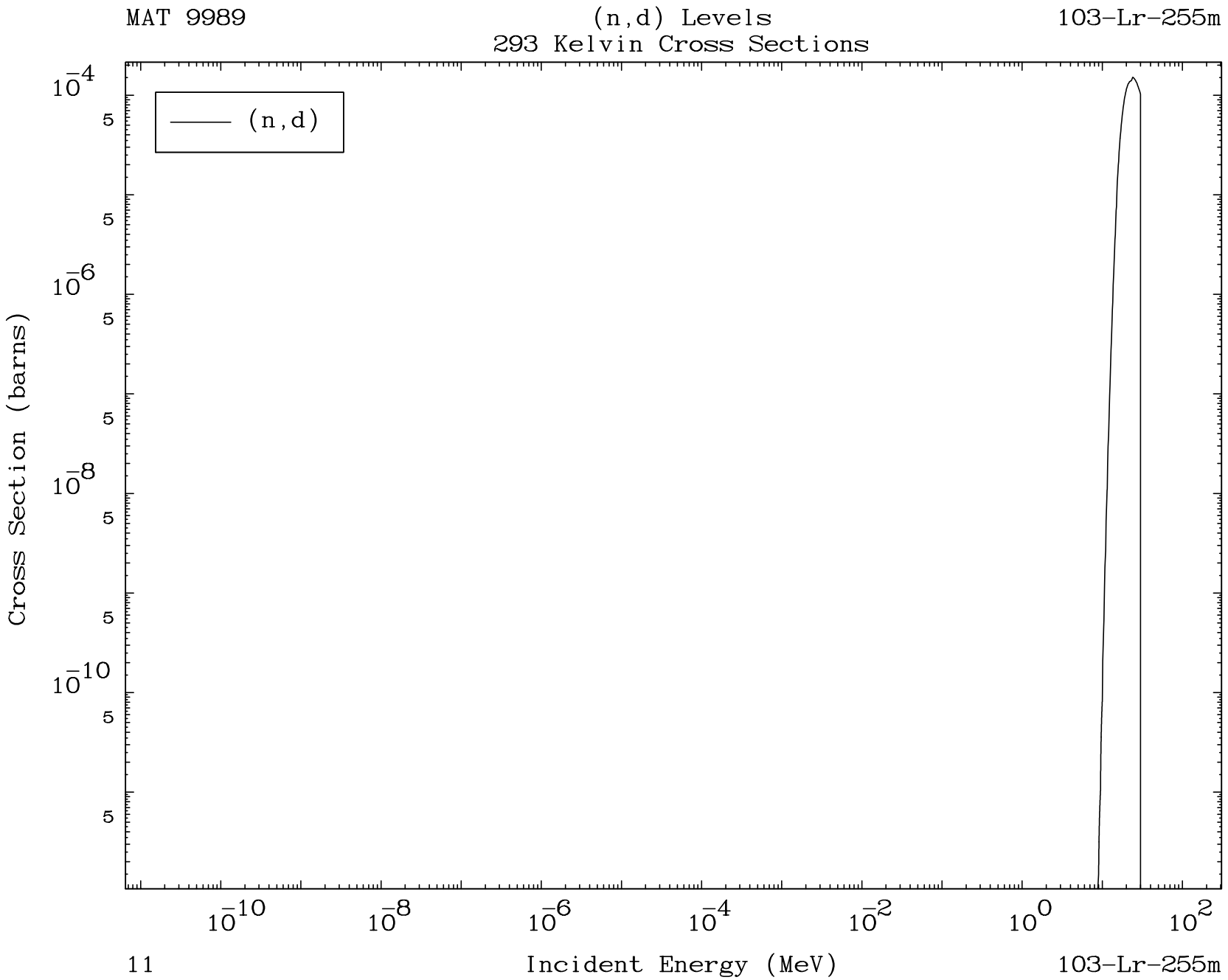

MAT 9989
Charged Particle
293 Kelvin Cross Sections
103-Lr-255m
Cross Section (barns)
Incident Energy (MeV)
103-Lr-255m
6
(n,n') α
(n,d)
(n,t)
(n,He-3)
(n,α)
(n,2p)
(n,p) α
(n,p) d

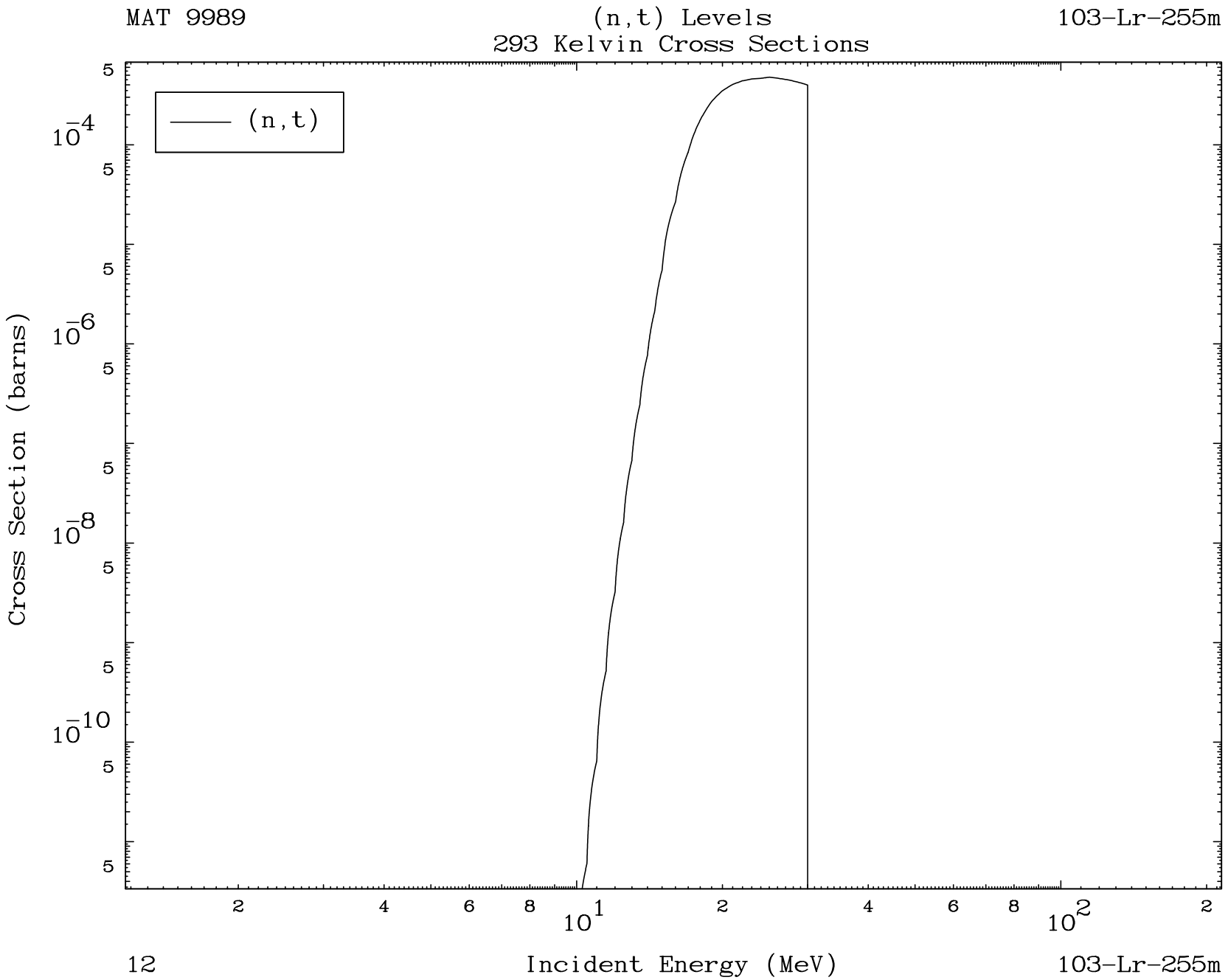

MAT 9989
Particle Production
293 Kelvin Cross Sections
103-Lr-255m
Hydrogen Production
Deuterium Production
Tritium Production
He-3 Production
He-4 Production
Cross Section (barns)
Incident Energy (MeV)
7
103-Lr-255m

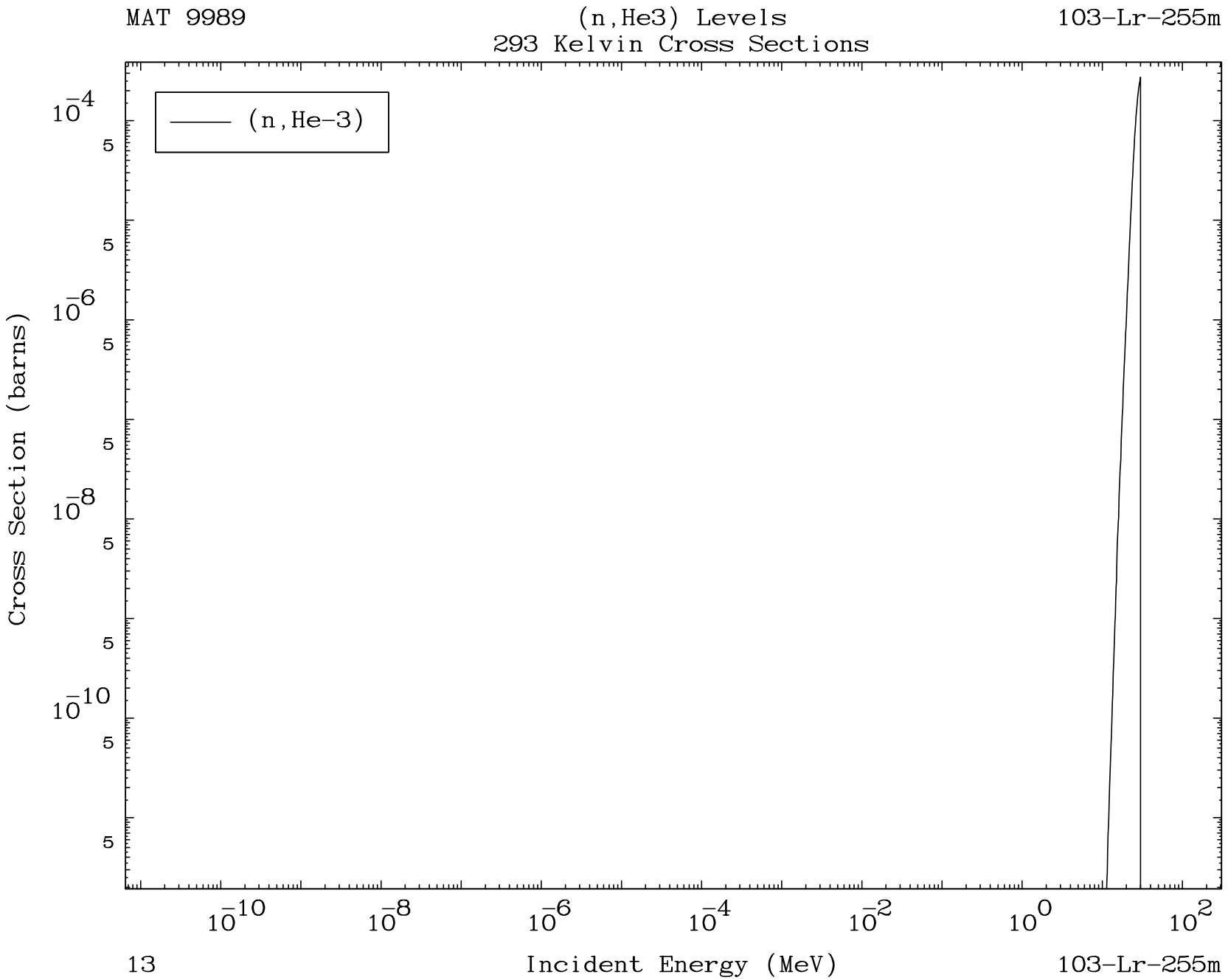

MAT 9989
Fission
103-Lr-255m
293 Kelvin Cross Sections
Cross Section (barns)
Incident Energy (MeV)
Fission
(n,f) First Chance
(n,nf) Second Chance
(n,2nf) Third Chance
(n,3nf) Fourth Chance
8
103-Lr-255m

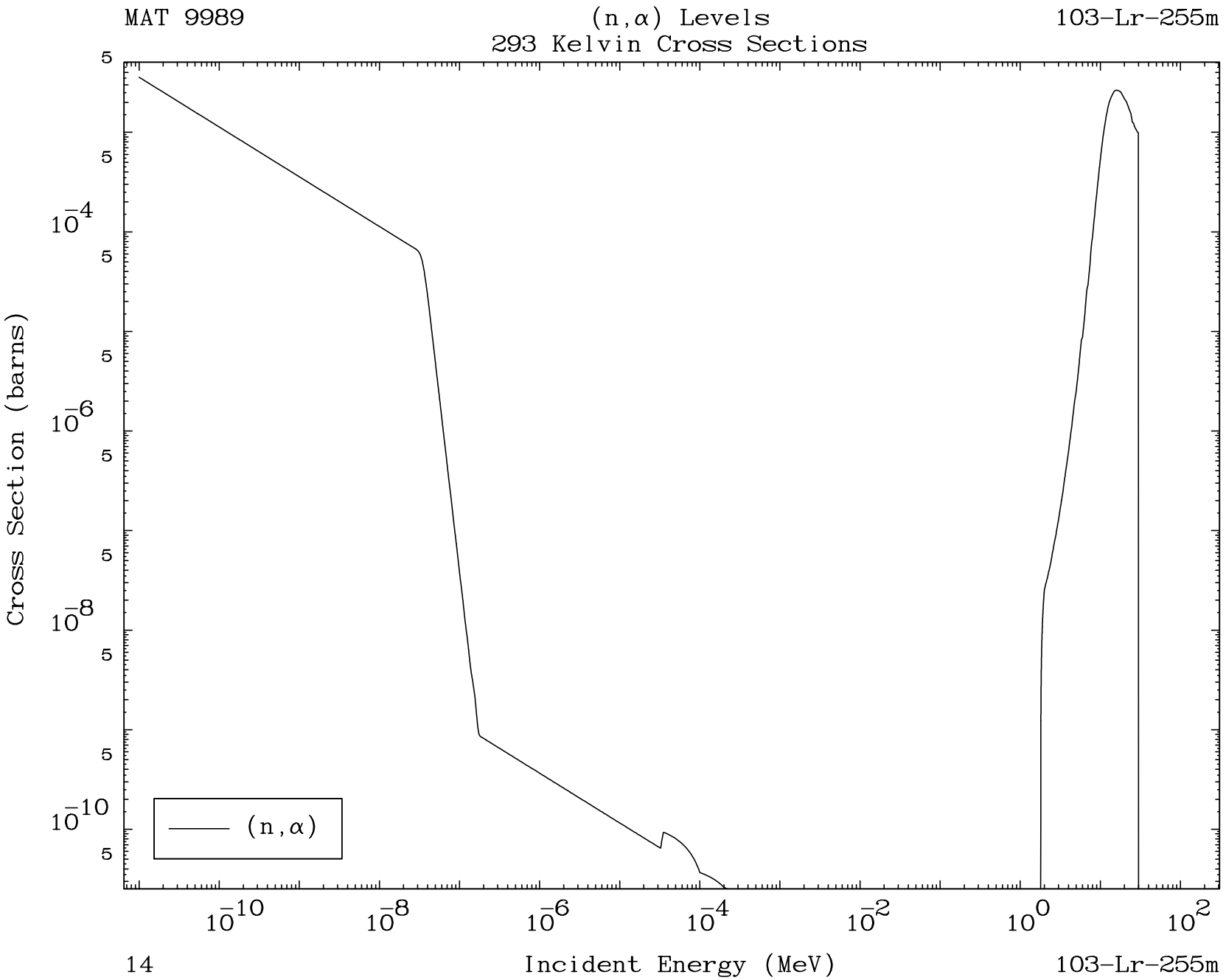

MAT 9989
(n,n') Levels
293 Kelvin Cross Sections
103-Lr-255m
Inelastic
MT= 51 (n,n') Level
MT= 52 (n,n') Level
(n,n') Continuum
Cross Section (barns)
Incident Energy (MeV)
9
103-Lr-255m

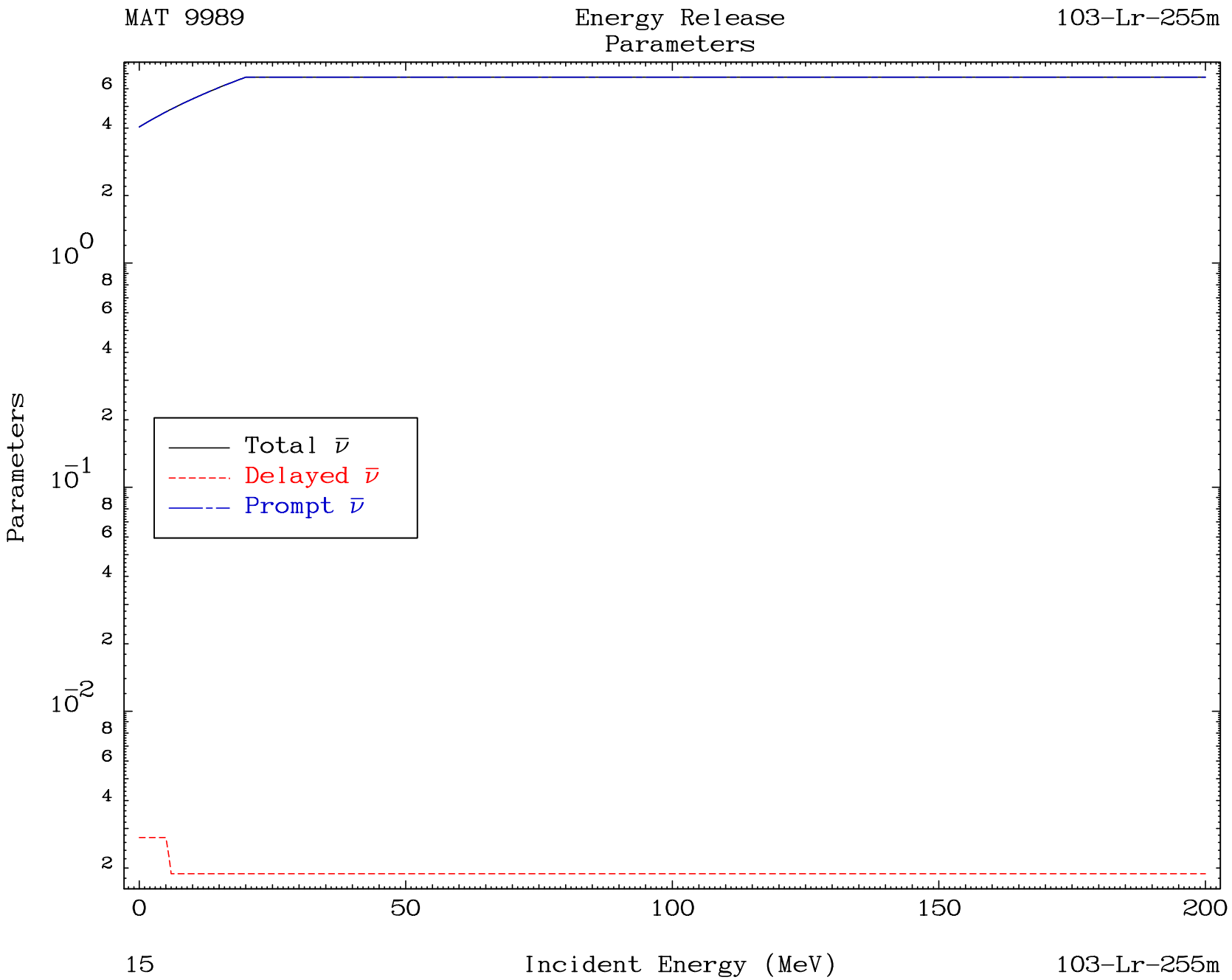

MAT 9989
(n,p) Levels
103-Lr-255m
293 Kelvin Cross Sections
Cross Section (barns)
Incident Energy (MeV)
(n,p)
103-Lr-255m

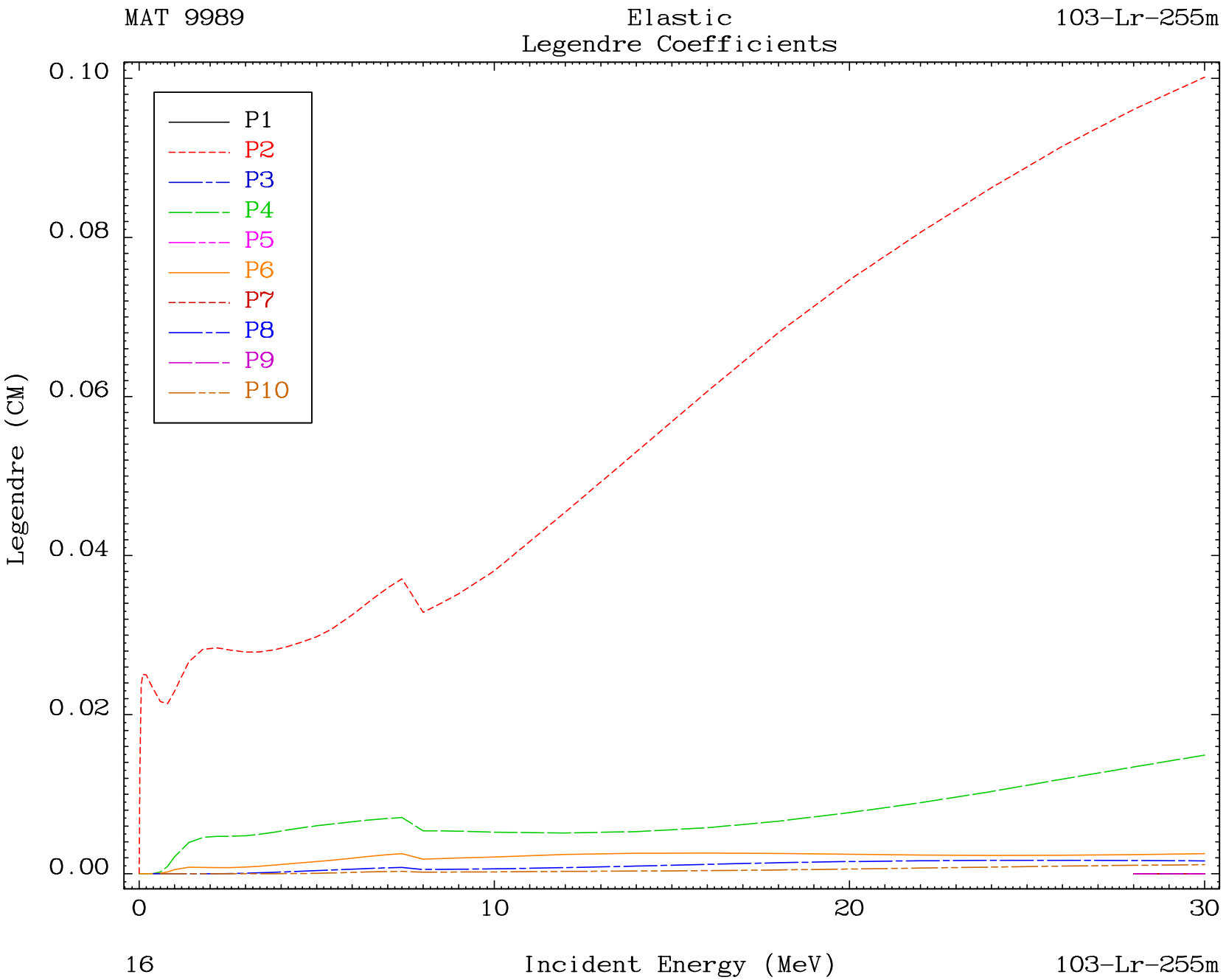

MAT 9989
(n,d) Levels
103-Lr-255m
293 Kelvin Cross Sections
(n,d)
Cross Section (barns)
Incident Energy (MeV)
11
103-Lr-255m

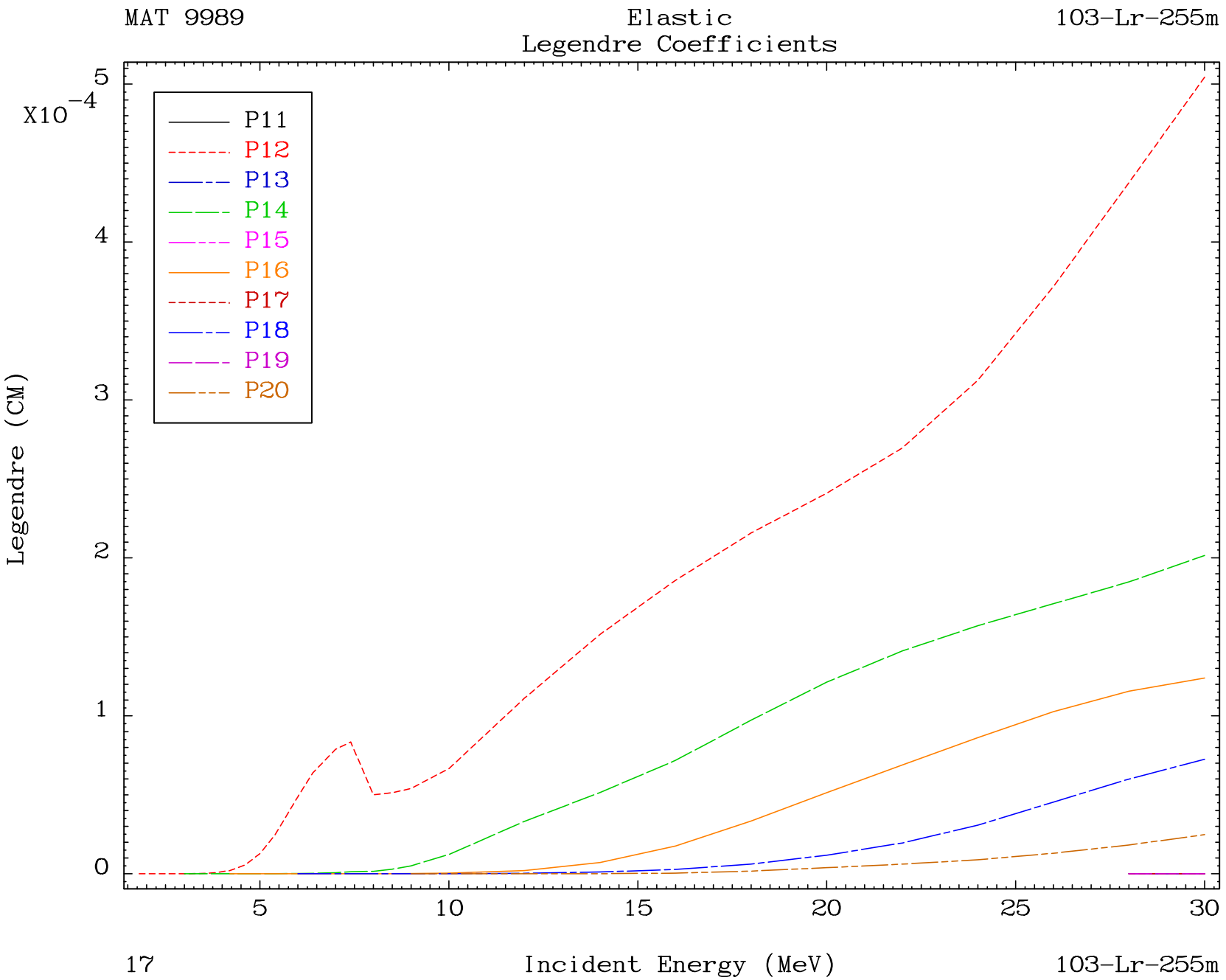

MAT 9989
(n,t) Levels
103-Lr-255m
293 Kelvin Cross Sections
(n,t)
Cross Section (barns)
Incident Energy (MeV)
12
103-Lr-255m

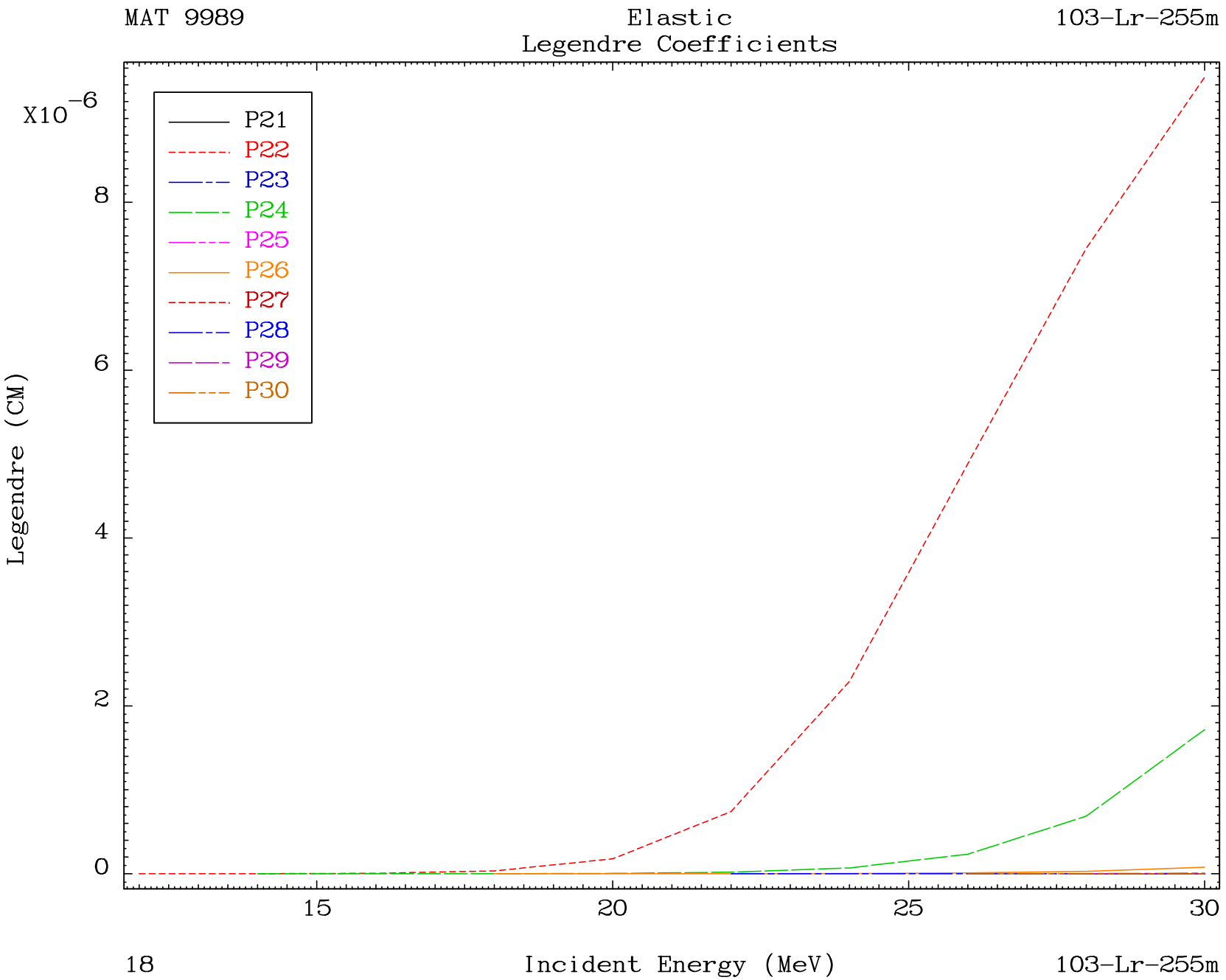

MAT 9989
(n,He3) Levels
103-Lr-255m
293 Kelvin Cross Sections
(n,He-3)
Cross Section (barns)
10^-4
10^-6
10^-8
10^-10
5
10^-10
10^-8
10^-6
10^-4
10^-2
10^0
10^2
13
Incident Energy (MeV)
103-Lr-255m

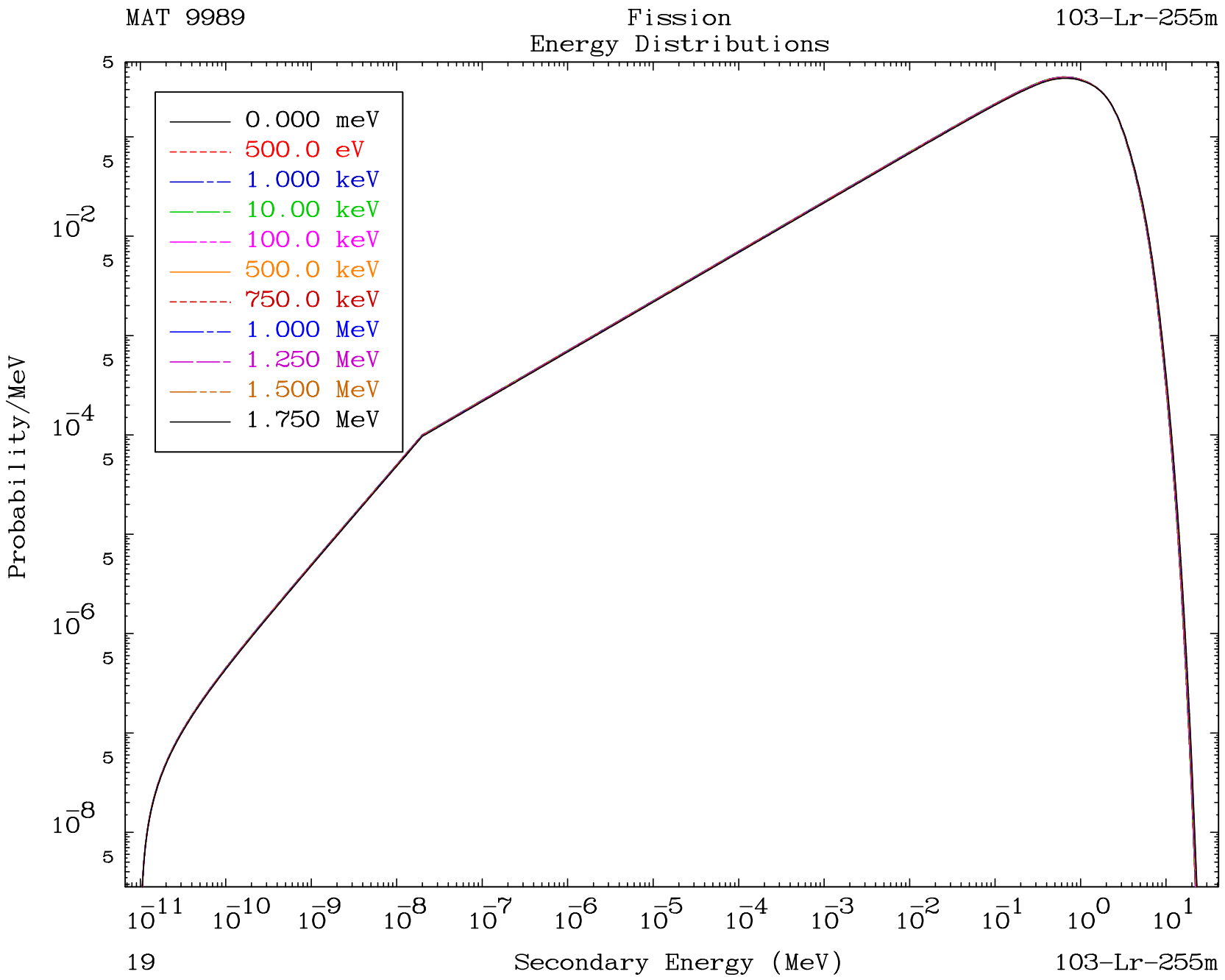

MAT 9989
(n,α) Levels
103-Lr-255m
293 Kelvin Cross Sections
Cross Section (barns)
5
5
10^-4
5
10^-6
5
10^-8
5
10^-10
5
(n,α)
10^-10
10^-8
10^-6
10^-4
10^-2
10^0
10^2
Incident Energy (MeV)
14
103-Lr-255m

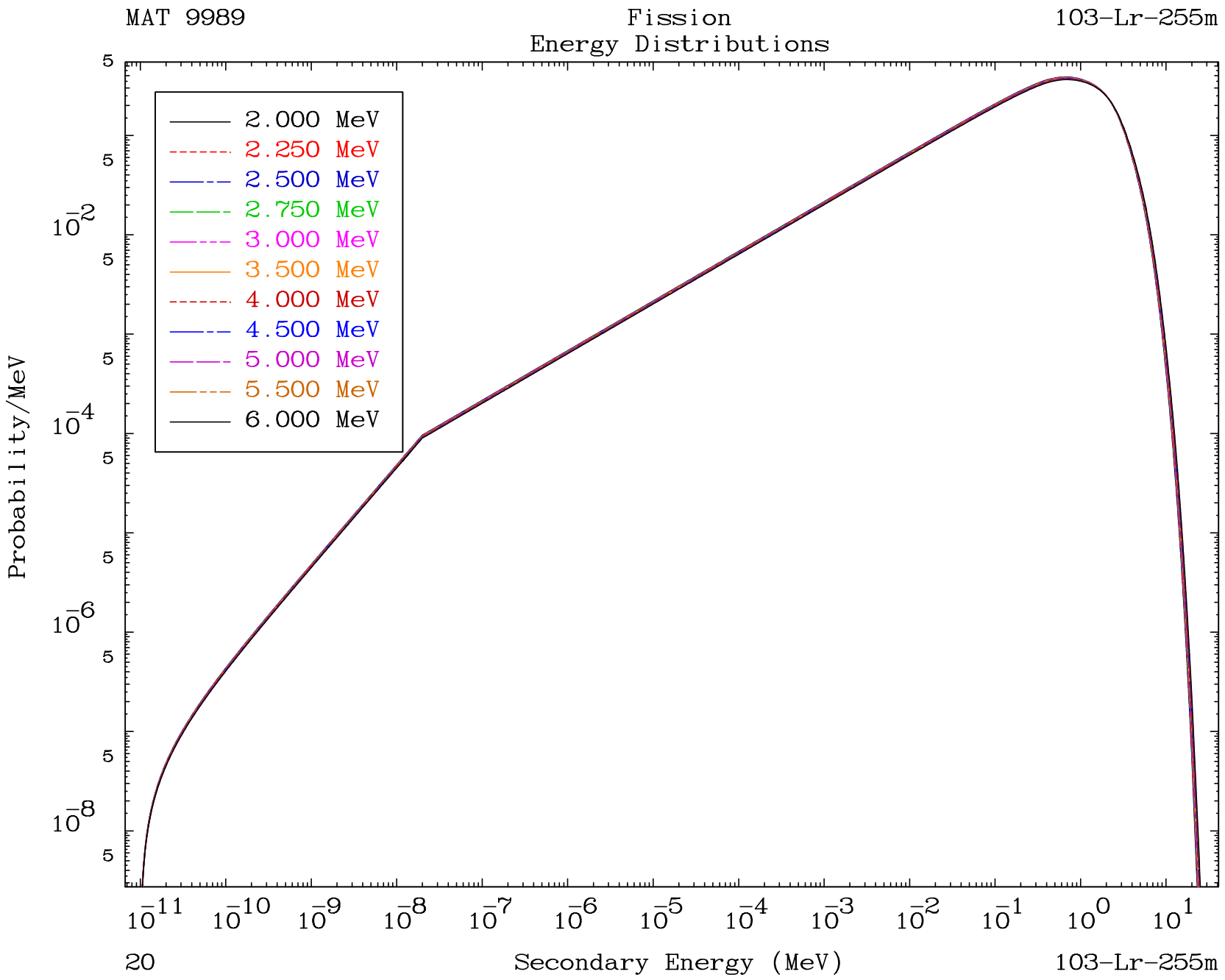

MAT 9989
Energy Release
Parameters
103-Lr-255m
Parameters
Total $\bar{\nu}$
Delayed $\bar{\nu}$
Prompt $\bar{\nu}$
Incident Energy (MeV)
15
103-Lr-255m

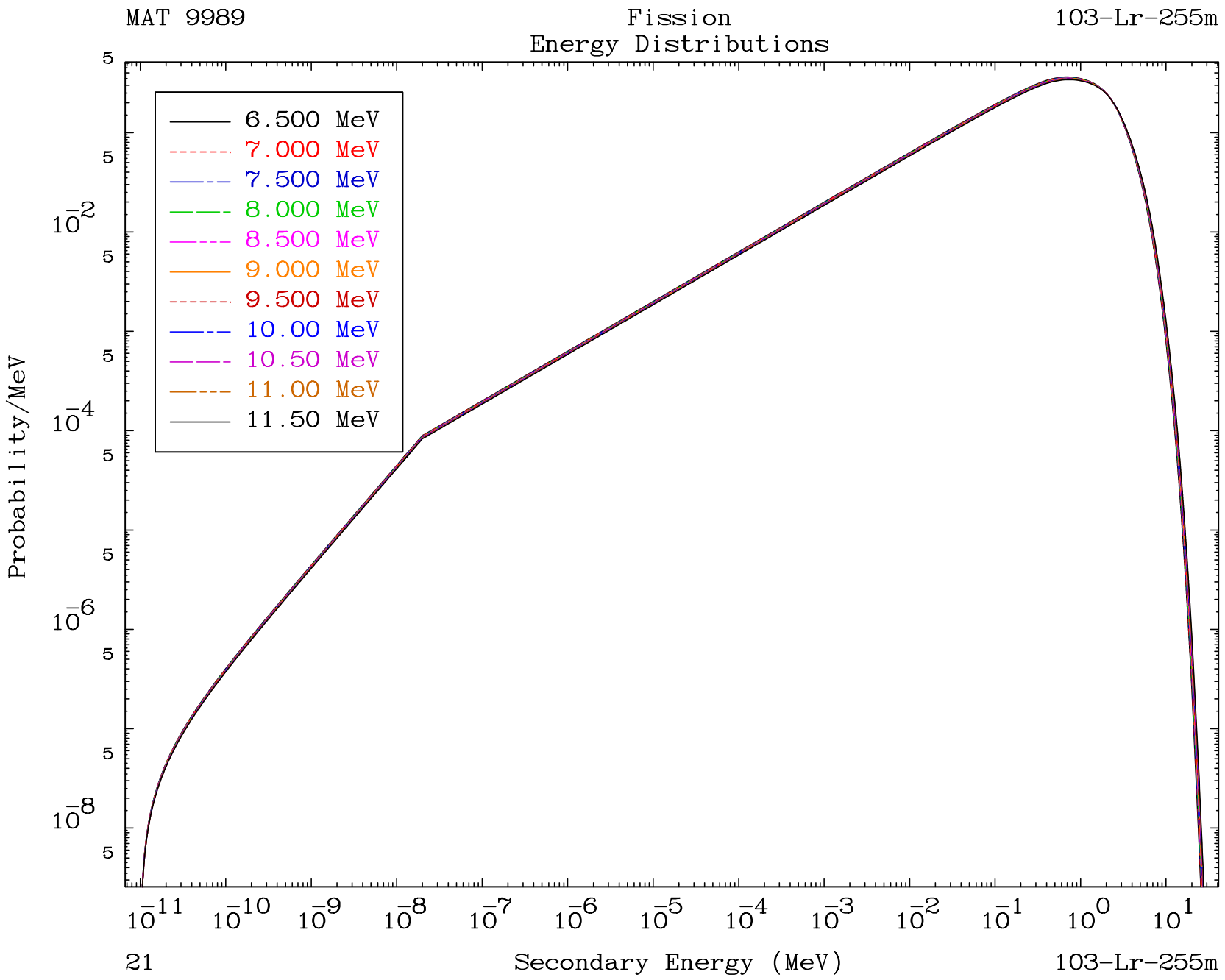

MAT 9989
Elastic
Legendre Coefficients
103-Lr-255m
0.10
0.08
0.06
0.04
0.02
0.00
Legendre (CM)
P1
P2
P3
P4
P5
P6
P7
P8
P9
P10
0
10
20
30
16
Incident Energy (MeV)
103-Lr-255m

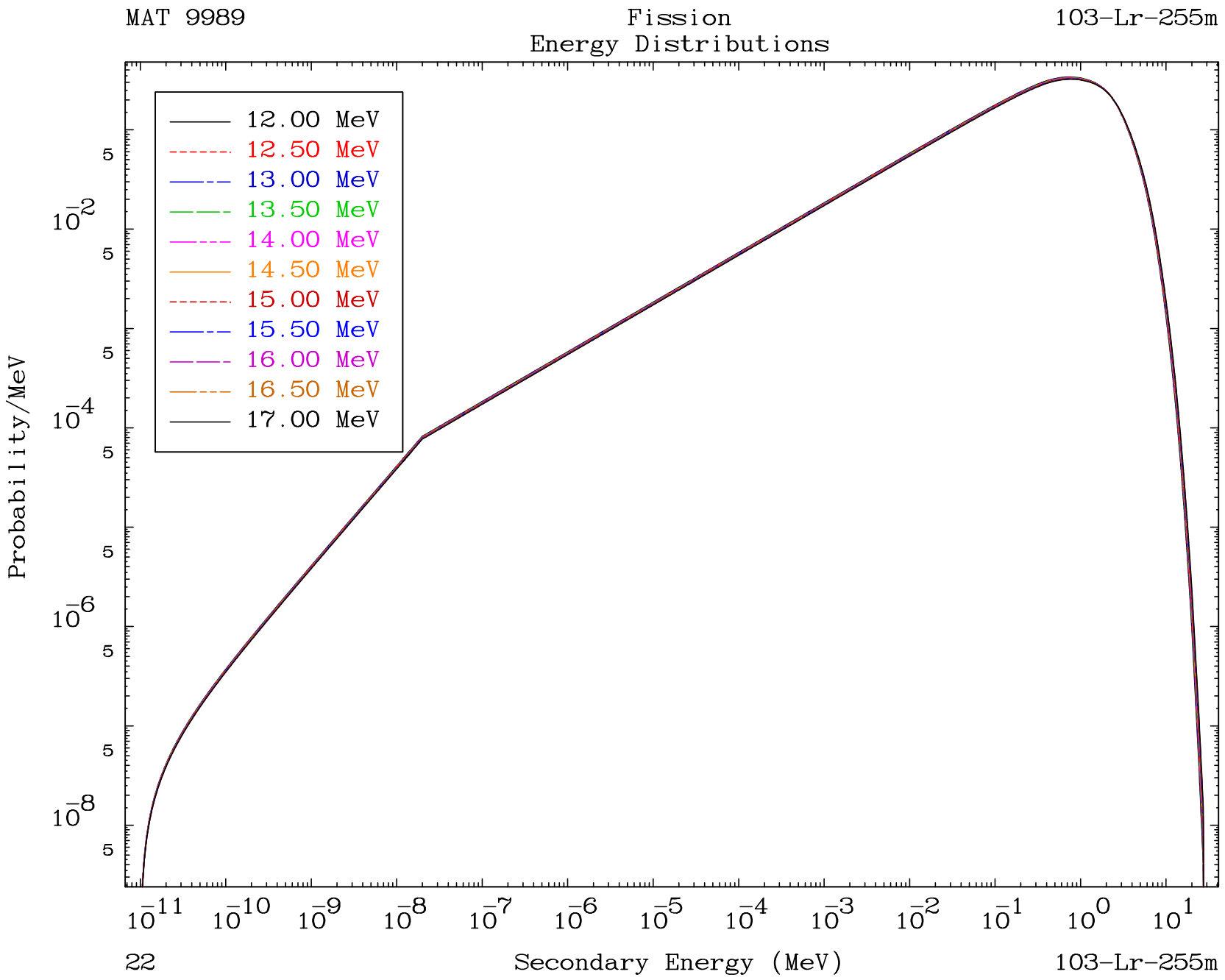

MAT 9989
Elastic
Legendre Coefficients
103-Lr-255m
X10 -4
5
4
3
2
1
0
Legendre (CM)
P11
P12
P13
P14
P15
P16
P17
P18
P19
P20
5
10
15
20
25
30
17
Incident Energy (MeV)
103-Lr-255m

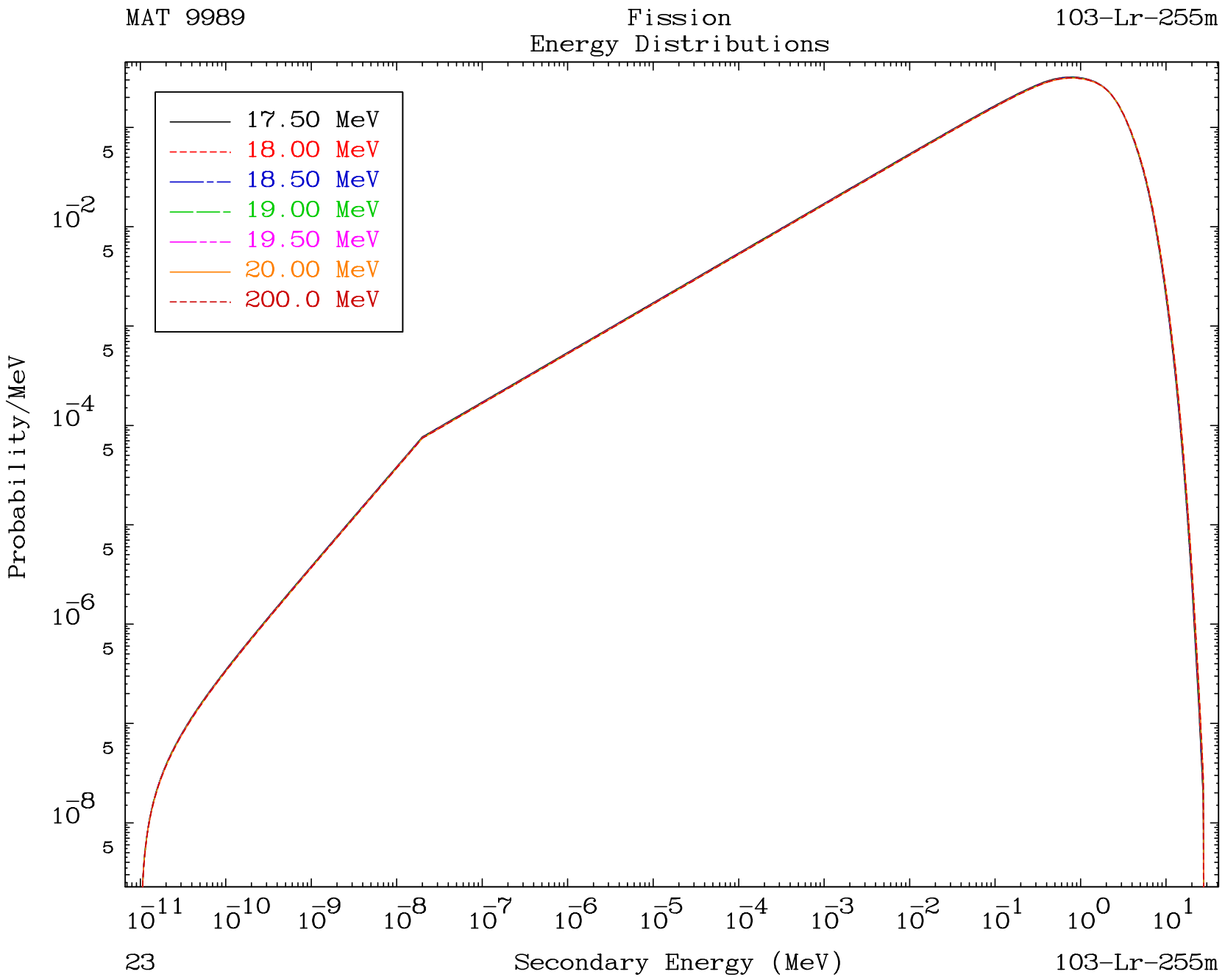

MAT 9989
Elastic
Legendre Coefficients
103-Lr-255m
X10 −6
8
6
4
2
0
Legendre (CM)
P21
P22
P23
P24
P25
P26
P27
P28
P29
P30
15
20
25
30
18
Incident Energy (MeV)
103-Lr-255m

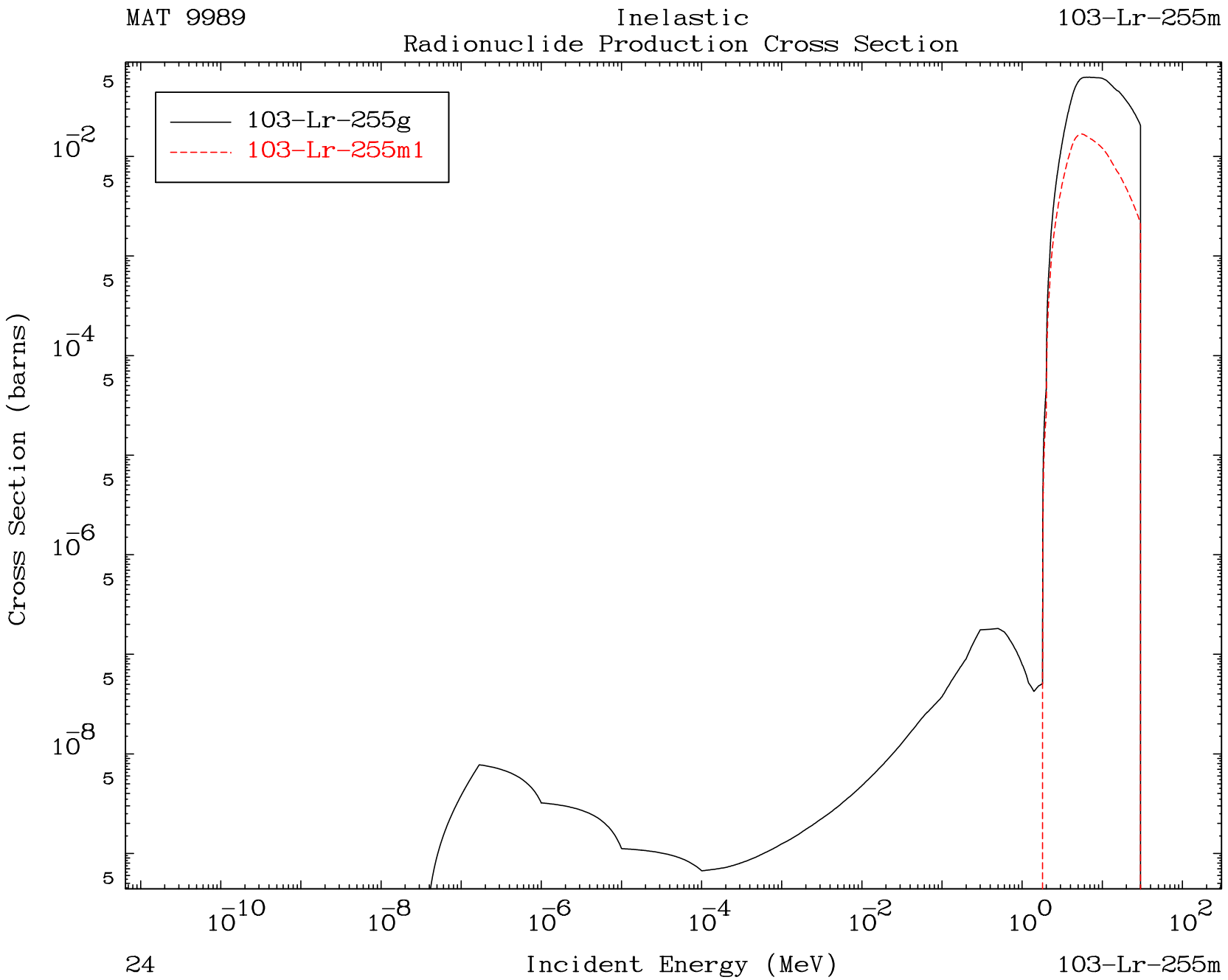

MAT 9989
Fission
Energy Distributions
103-Lr-255m
0.000 meV
500.0 eV
1.000 keV
10.00 keV
100.0 keV
500.0 keV
750.0 keV
1.000 MeV
1.250 MeV
1.500 MeV
1.750 MeV
Probability/MeV
Secondary Energy (MeV)
19
103-Lr-255m

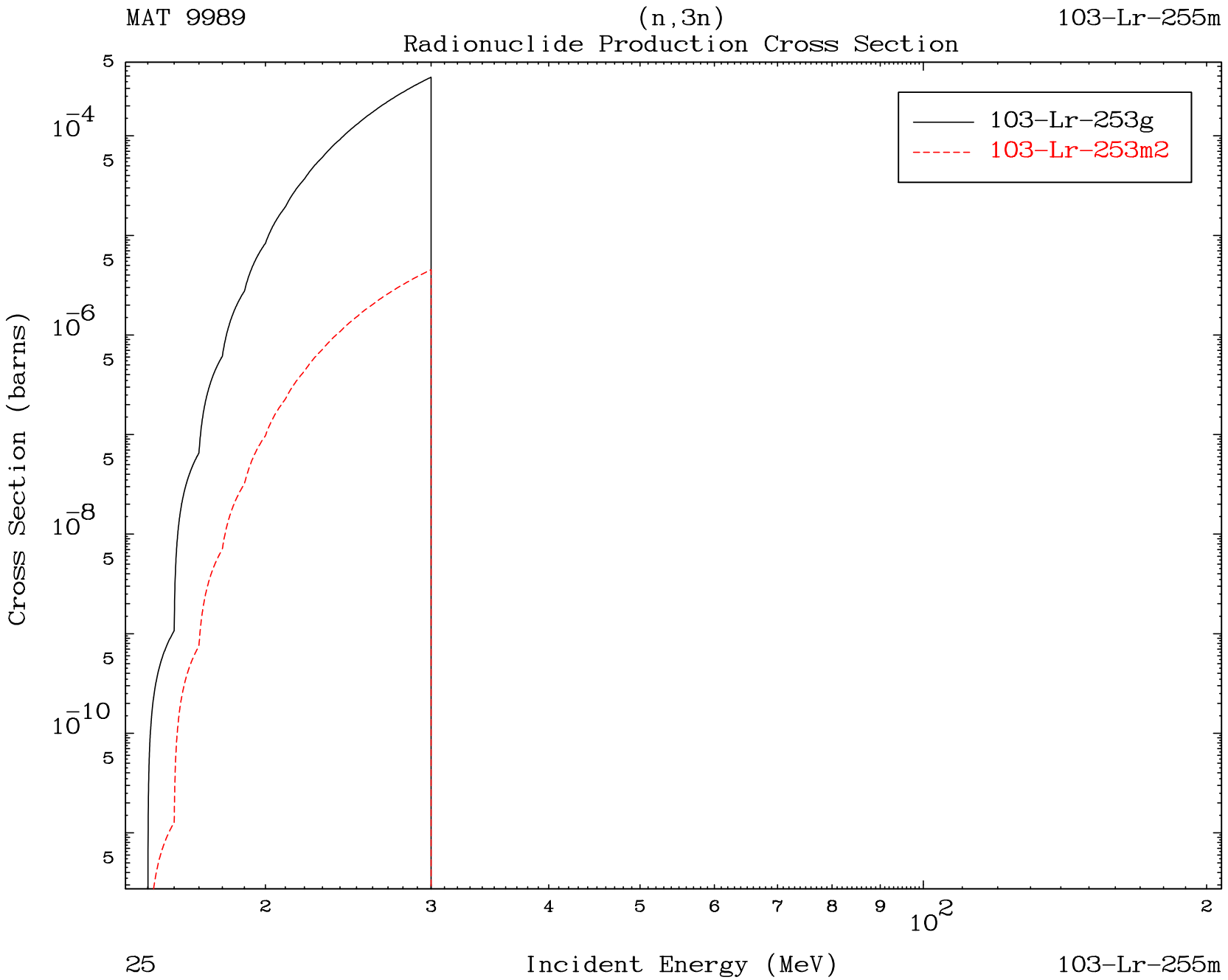

MAT 9989
Fission
Energy Distributions
103-Lr-255m
2.000 MeV
2.250 MeV
2.500 MeV
2.750 MeV
3.000 MeV
3.500 MeV
4.000 MeV
4.500 MeV
5.000 MeV
5.500 MeV
6.000 MeV
Probability/MeV
Secondary Energy (MeV)
20
103-Lr-255m

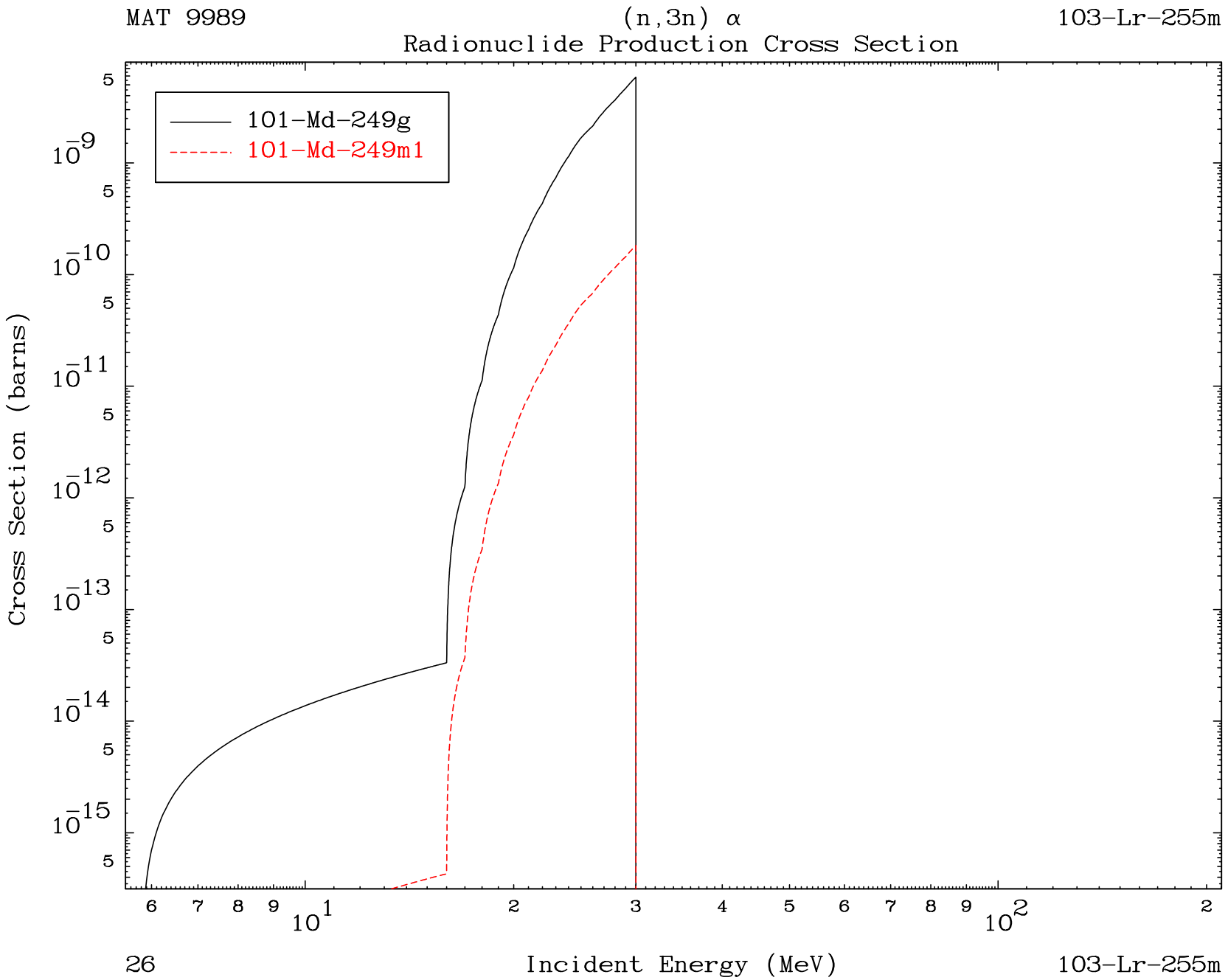

MAT 9989
Fission
Energy Distributions
103-Lr-255m
6.500 MeV
7.000 MeV
7.500 MeV
8.000 MeV
8.500 MeV
9.000 MeV
9.500 MeV
10.00 MeV
10.50 MeV
11.00 MeV
11.50 MeV
Probability/MeV
Secondary Energy (MeV)
21
103-Lr-255m

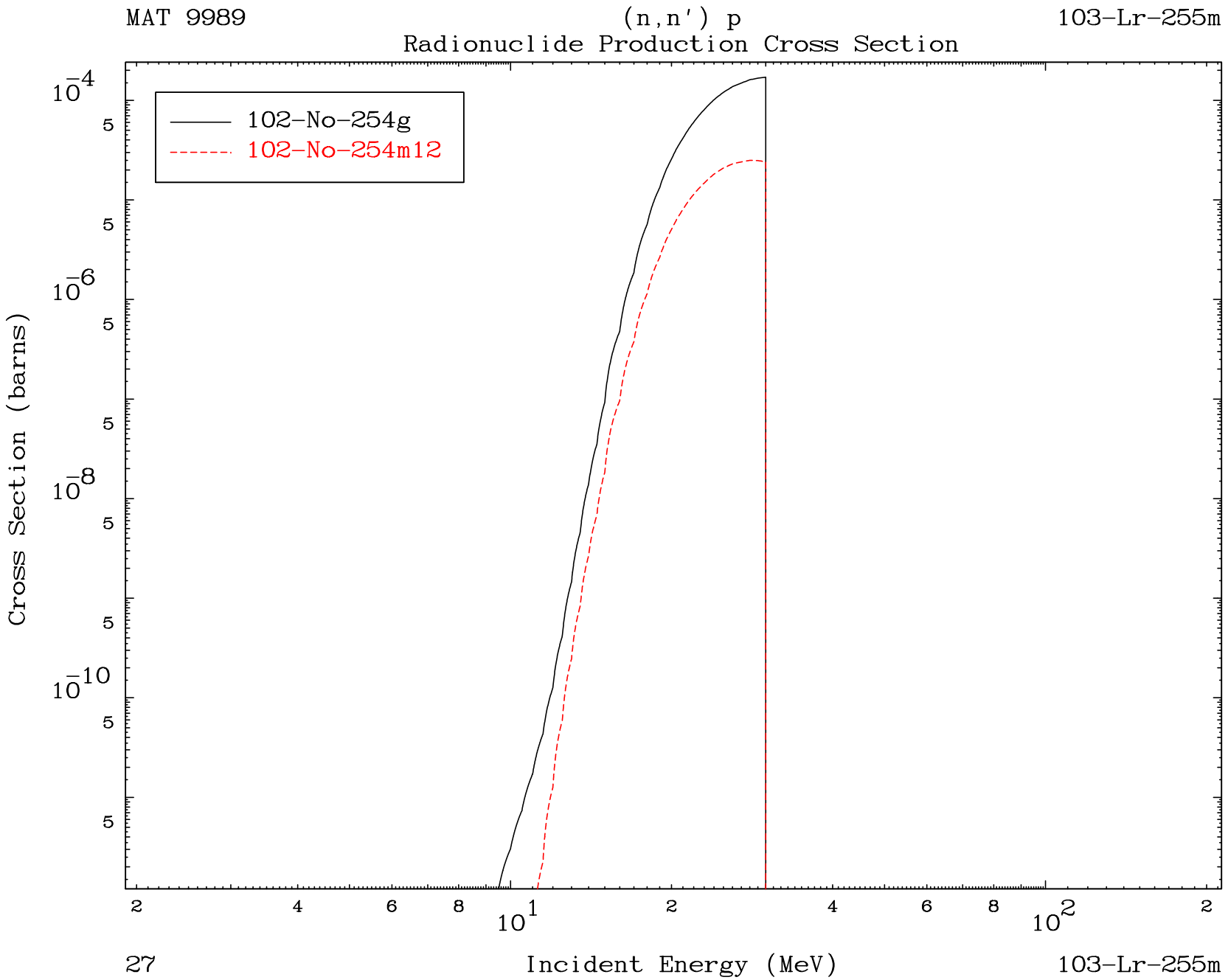

MAT 9989
Fission
Energy Distributions
103-Lr-255m
12.00 MeV
12.50 MeV
13.00 MeV
13.50 MeV
14.00 MeV
14.50 MeV
15.00 MeV
15.50 MeV
16.00 MeV
16.50 MeV
17.00 MeV
Probability/MeV
Secondary Energy (MeV)
22
103-Lr-255m

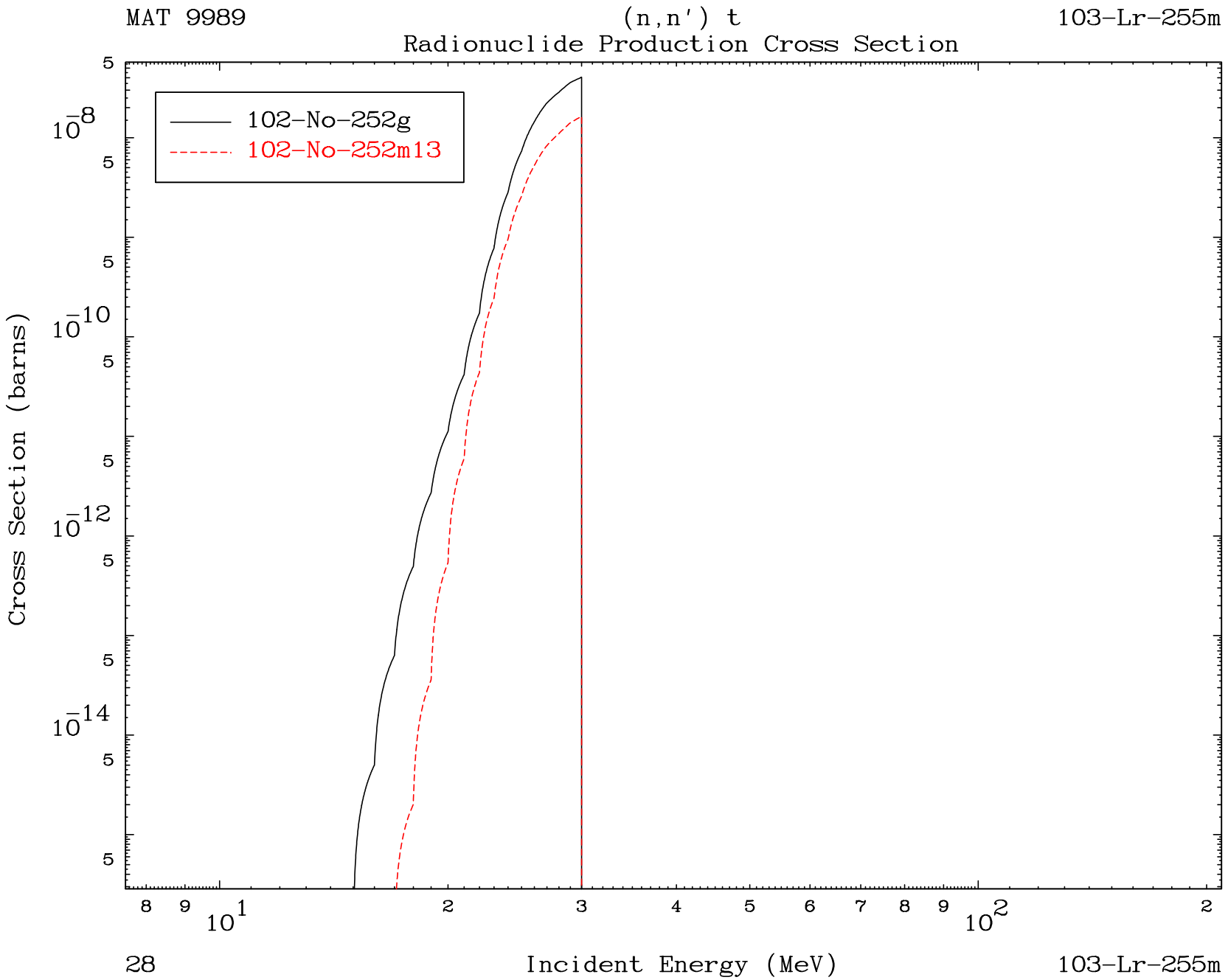

MAT 9989
Fission
Energy Distributions
103-Lr-255m
17.50 MeV
18.00 MeV
18.50 MeV
19.00 MeV
19.50 MeV
20.00 MeV
200.0 MeV
Probability/MeV
Secondary Energy (MeV)
23
103-Lr-255m

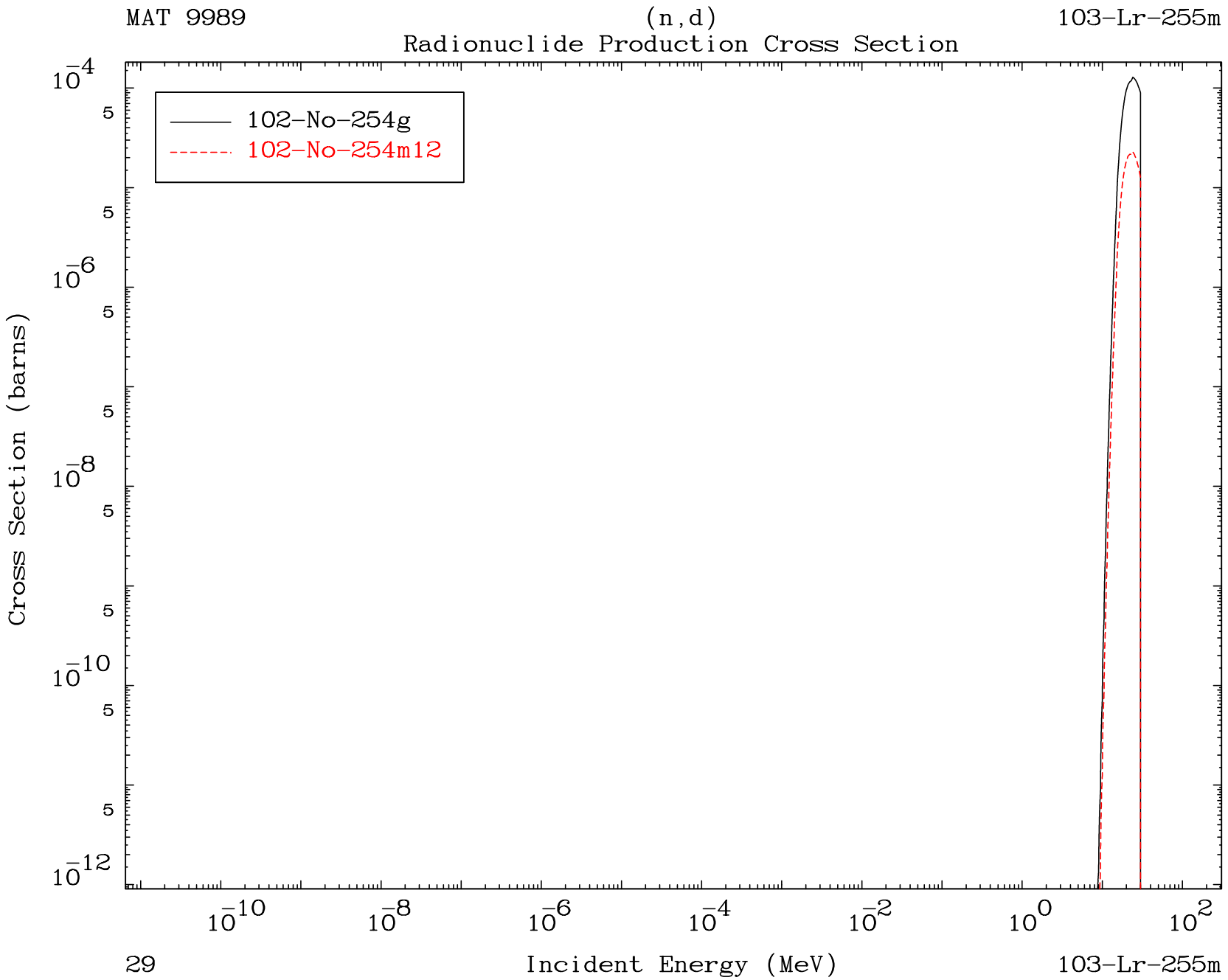

MAT 9989
Inelastic
103-Lr-255m
Radionuclide Production Cross Section
103-Lr-255g
103-Lr-255m1
Cross Section (barns)
Incident Energy (MeV)
24
103-Lr-255m

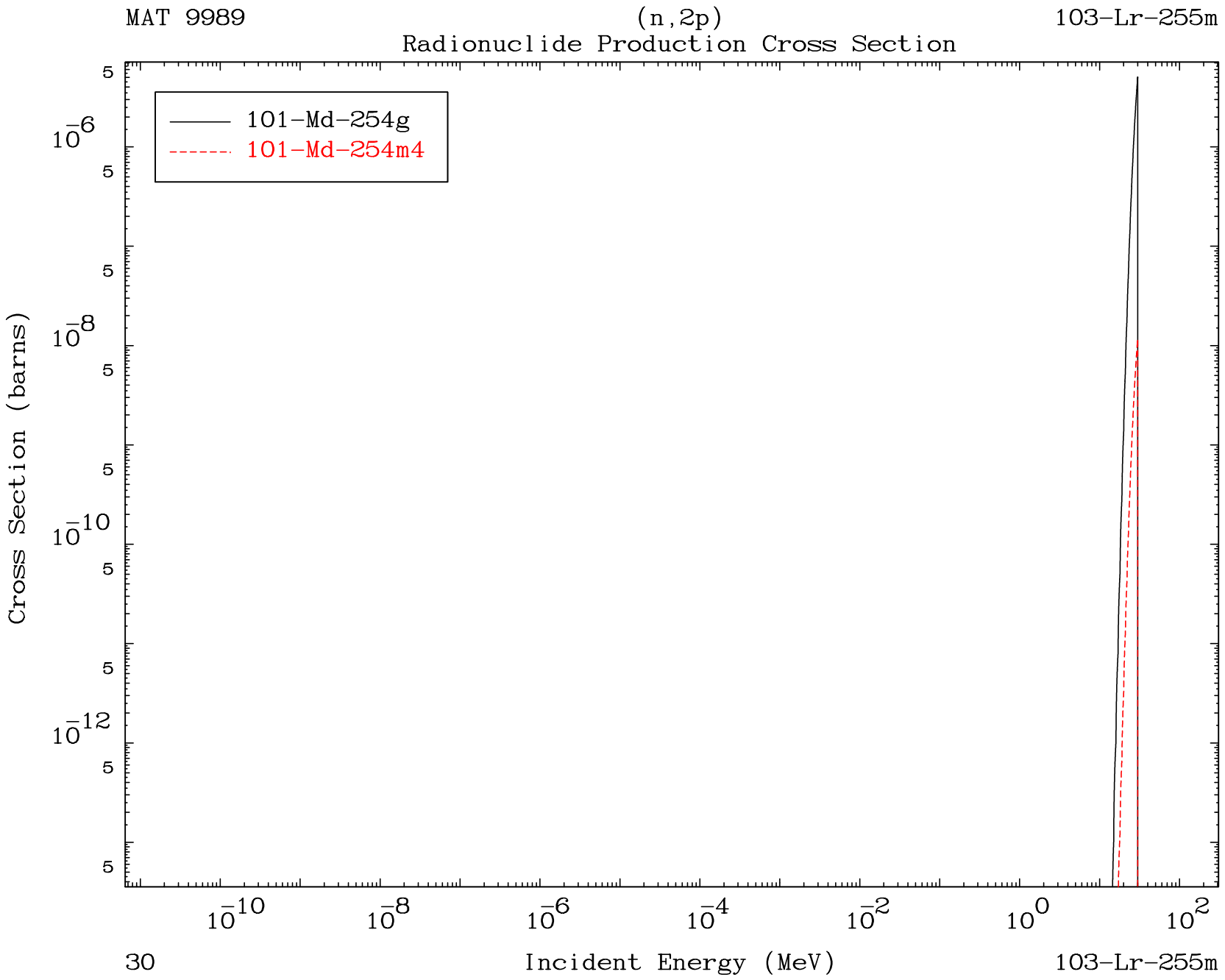

MAT 9989
(n,3n)
103-Lr-255m
Radionuclide Production Cross Section
103-Lr-253g
103-Lr-253m2
Cross Section (barns)
10^-4
10^-6
10^-8
10^-10
Incident Energy (MeV)
10^2
25
103-Lr-255m


MAT 9989
(n,3n) α
103-Lr-255m
Radionuclide Production Cross Section
101-Md-249g
101-Md-249m1
Cross Section (barns)
Incident Energy (MeV)
26
103-Lr-255m


MAT 9989
(n,n') p
103-Lr-255m
Radionuclide Production Cross Section
102-No-254g
102-No-254m12
Cross Section (barns)
Incident Energy (MeV)
27
103-Lr-255m


MAT 9989
(n,n') t
103-Lr-255m
Radionuclide Production Cross Section
102-No-252g
102-No-252m13
Cross Section (barns)
Incident Energy (MeV)
28
103-Lr-255m


MAT 9989
(n,d)
103-Lr-255m
Radionuclide Production Cross Section
102-No-254g
102-No-254m12
Cross Section (barns)
Incident Energy (MeV)
29
103-Lr-255m


MAT 9989
(n,2p)
103-Lr-255m
Radionuclide Production Cross Section
101-Md-254g
101-Md-254m4
Cross Section (barns)
Incident Energy (MeV)
30
103-Lr-255m