

Program EVALPLOT
(Version 2021-1)

by

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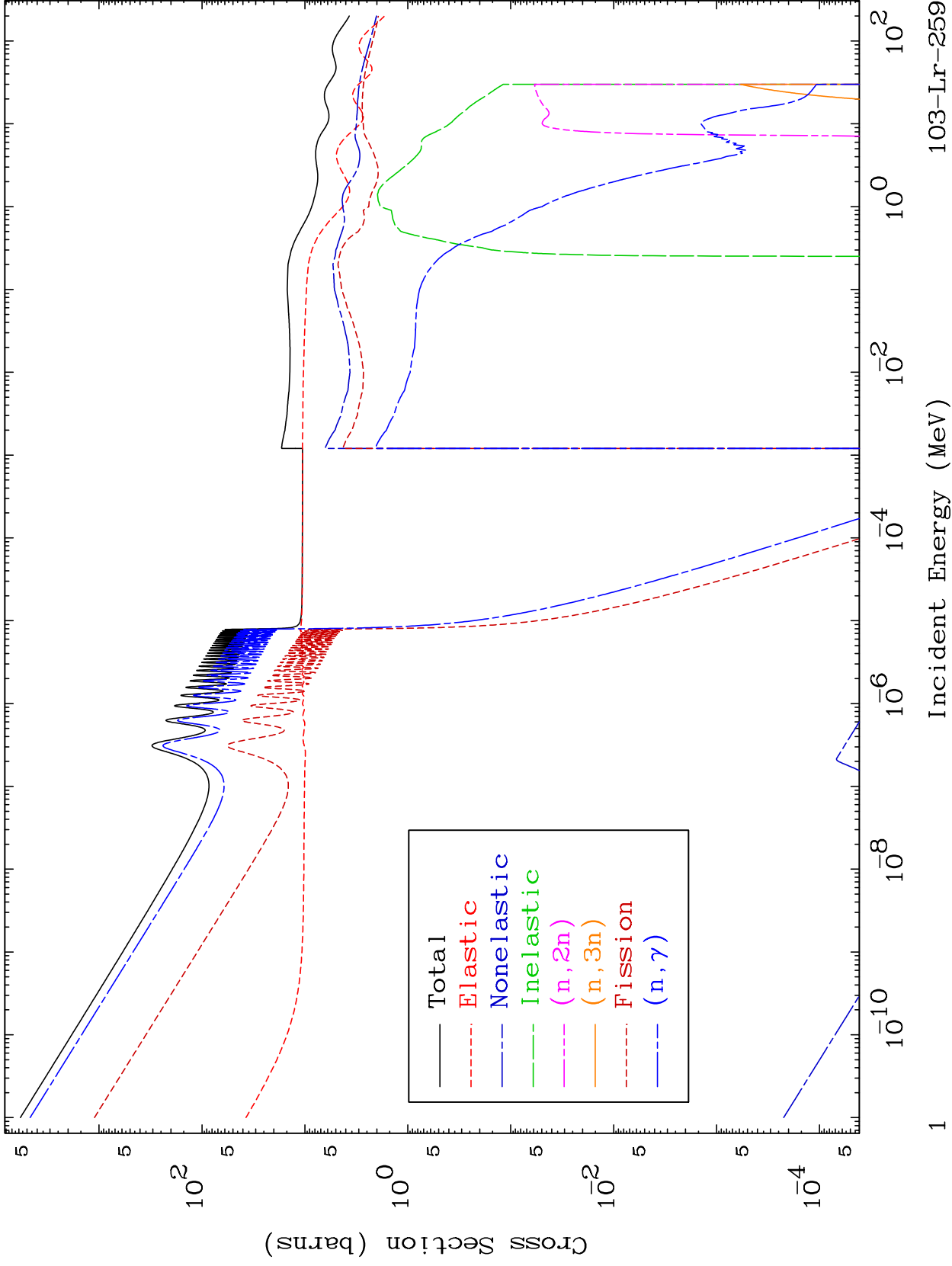
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Press Mouse Button to Start

MAT 9988

Neutron Major
293 Kelvin Cross Sections

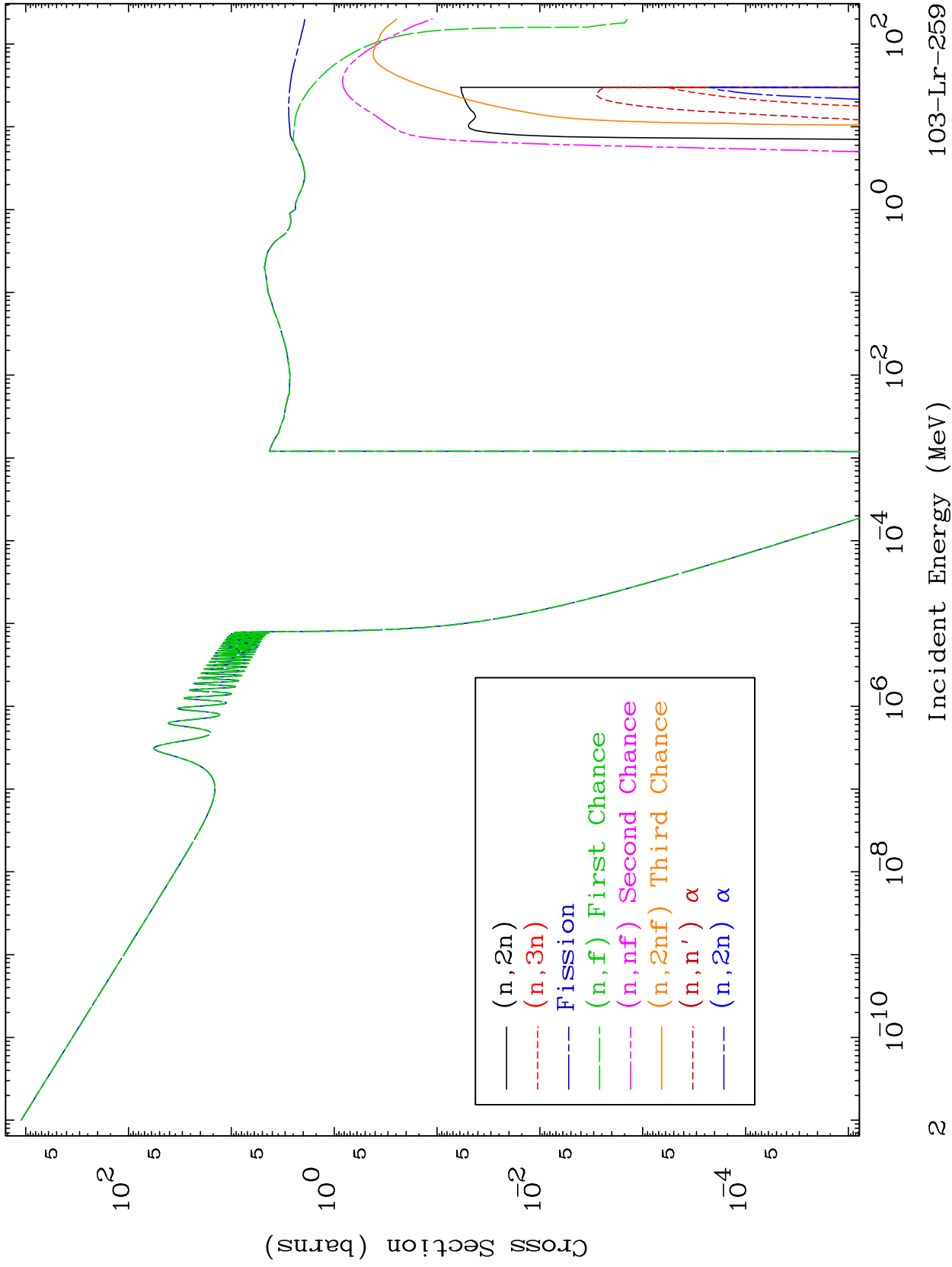
103-Lr-259



MAT 9988

Neutron Absorption
293 Kelvin Cross Sections

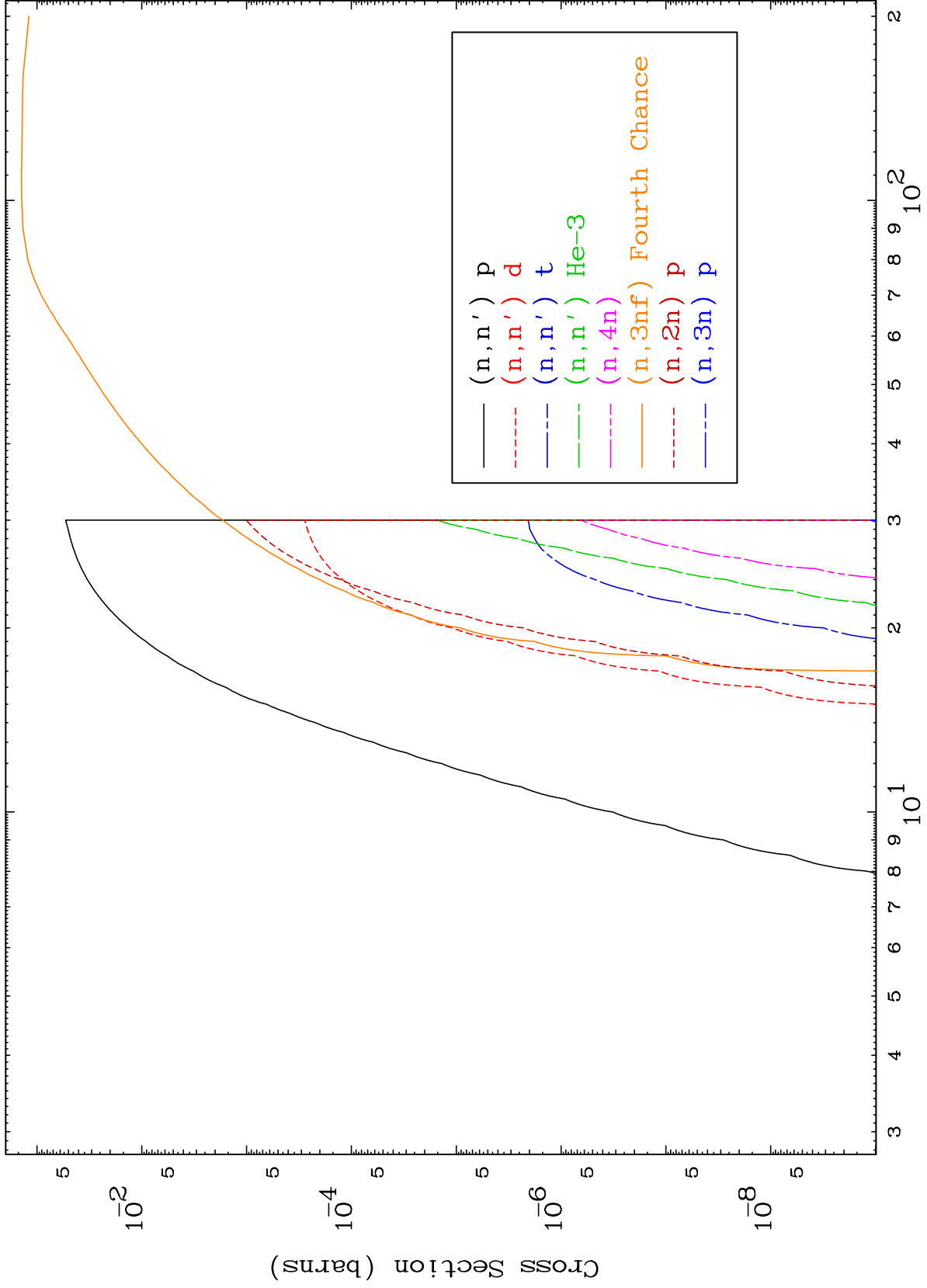
103-Lr-259



MAT 9988

Neutron Absorption
293 Kelvin Cross Sections

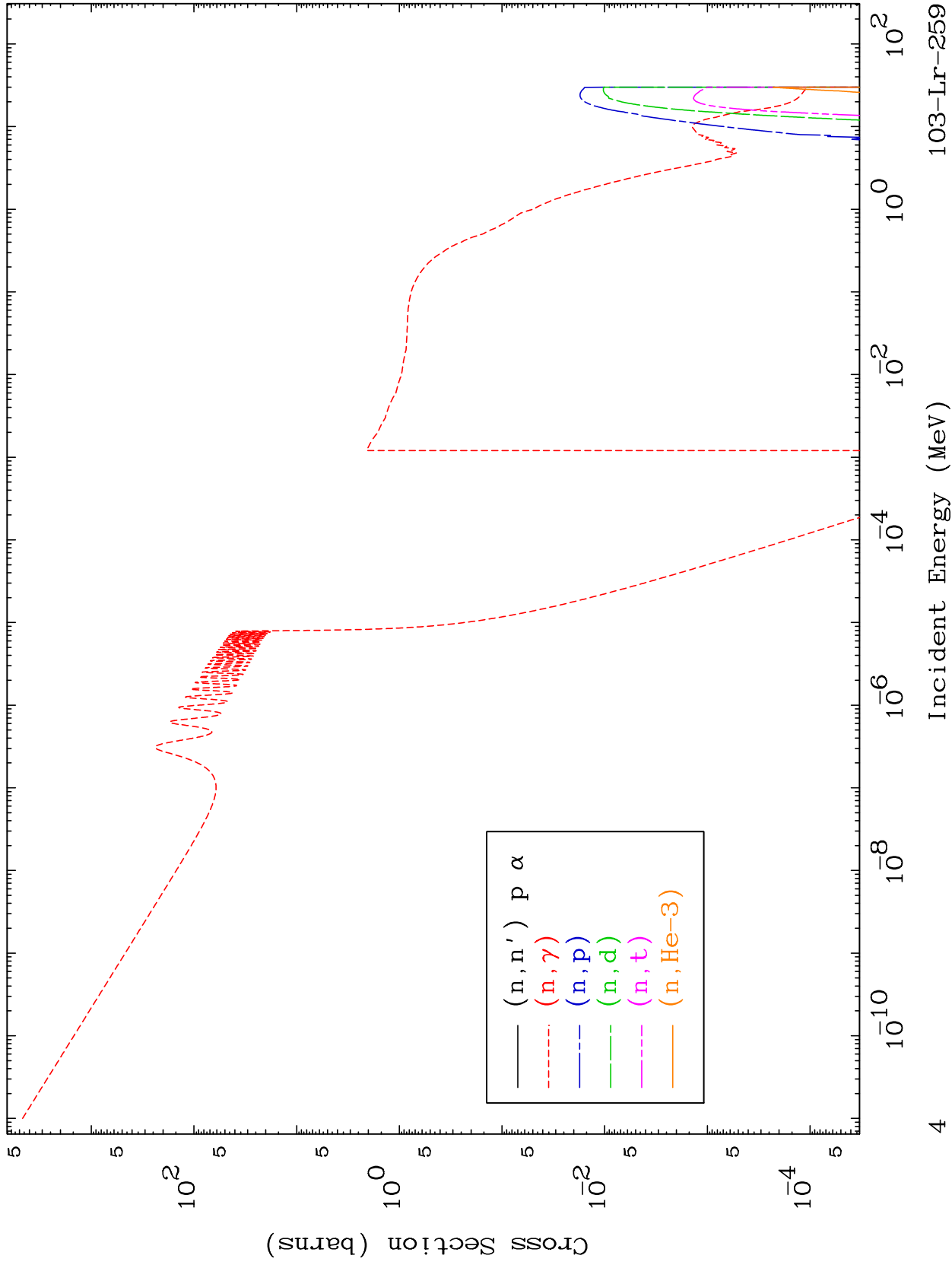
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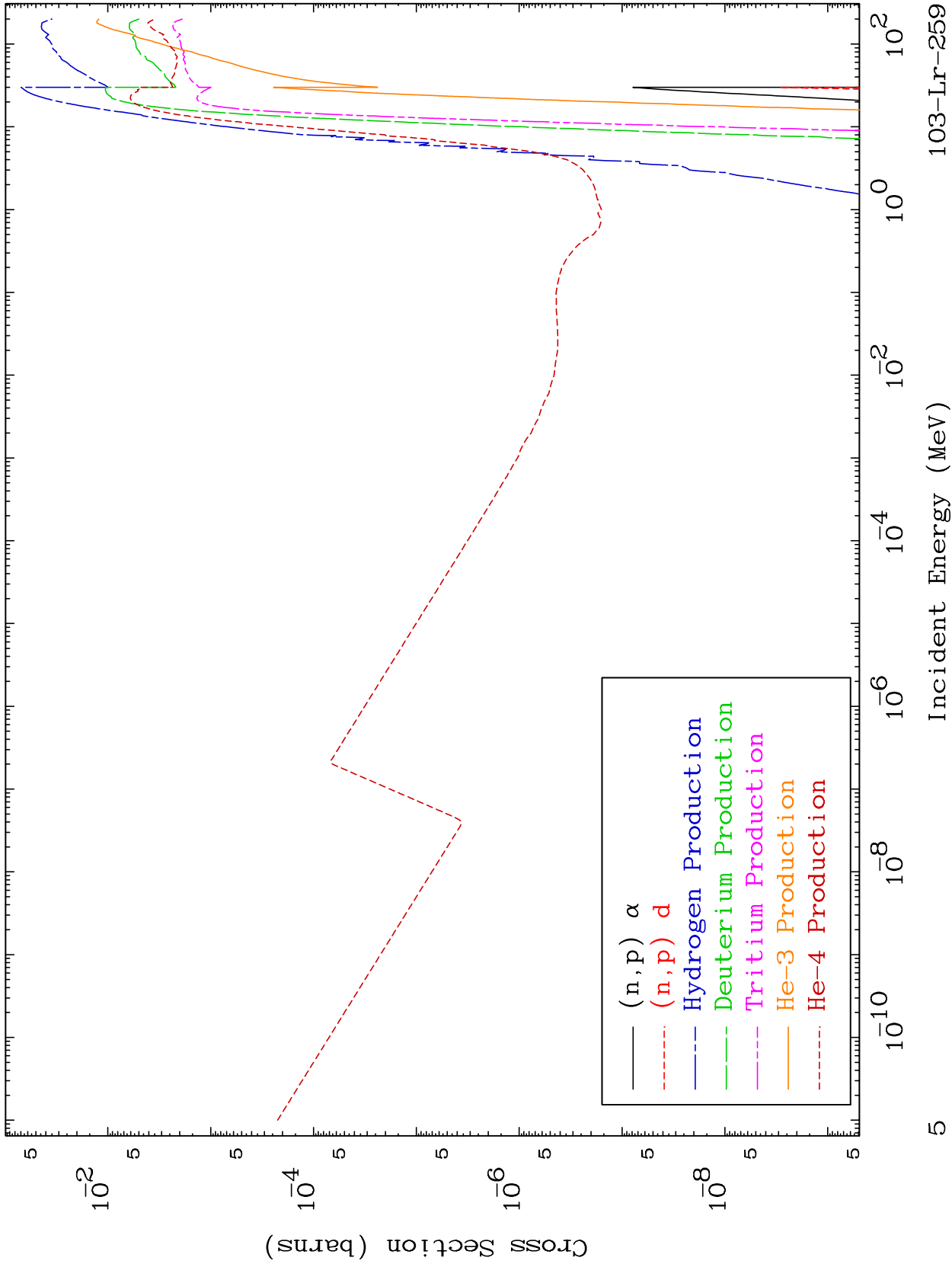


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Neutron Absorption
293 Kelvin Cross Sections

103-Lr-259

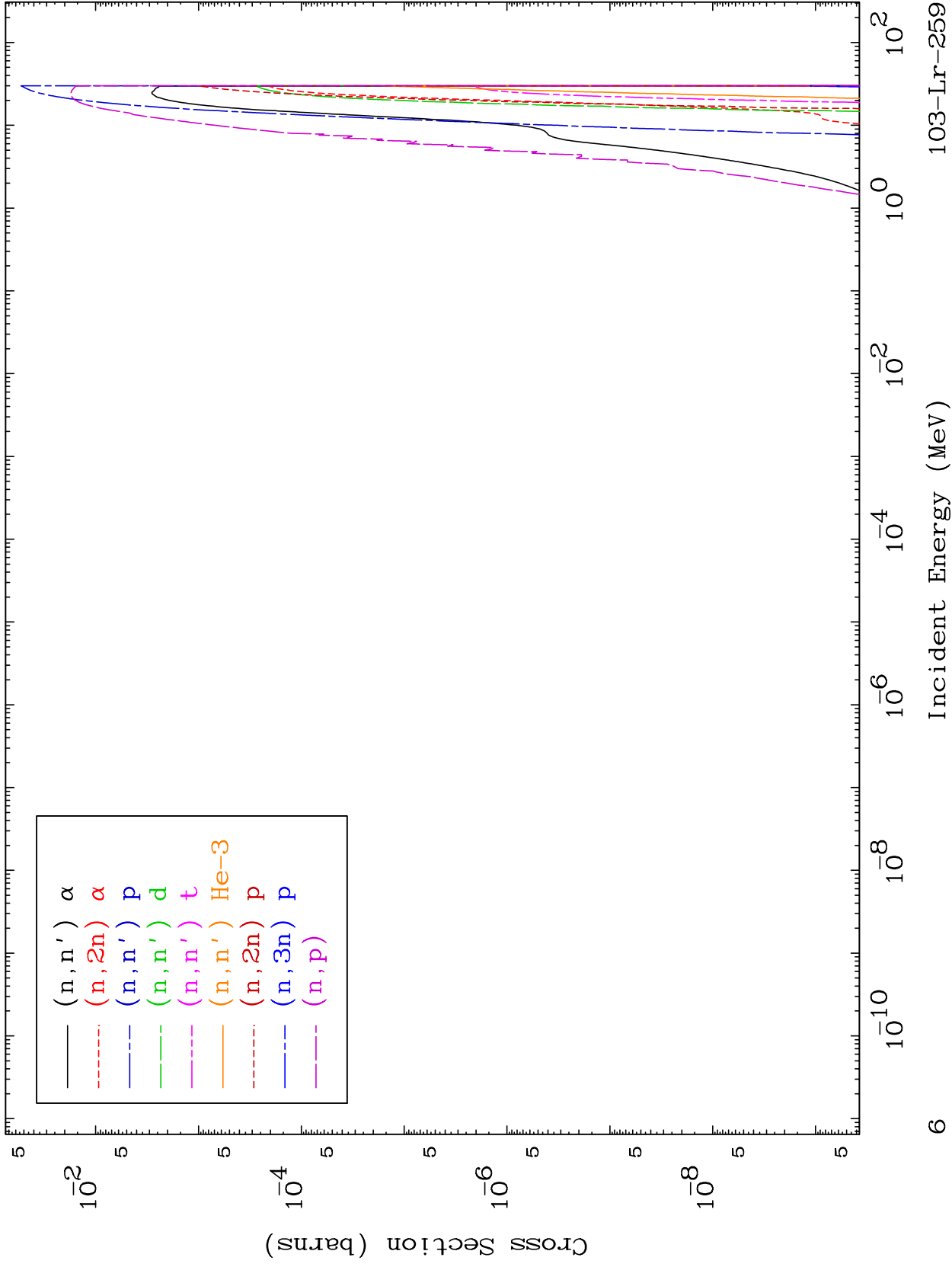




MAT 9988

Charged Particle
293 Kelvin Cross Sections

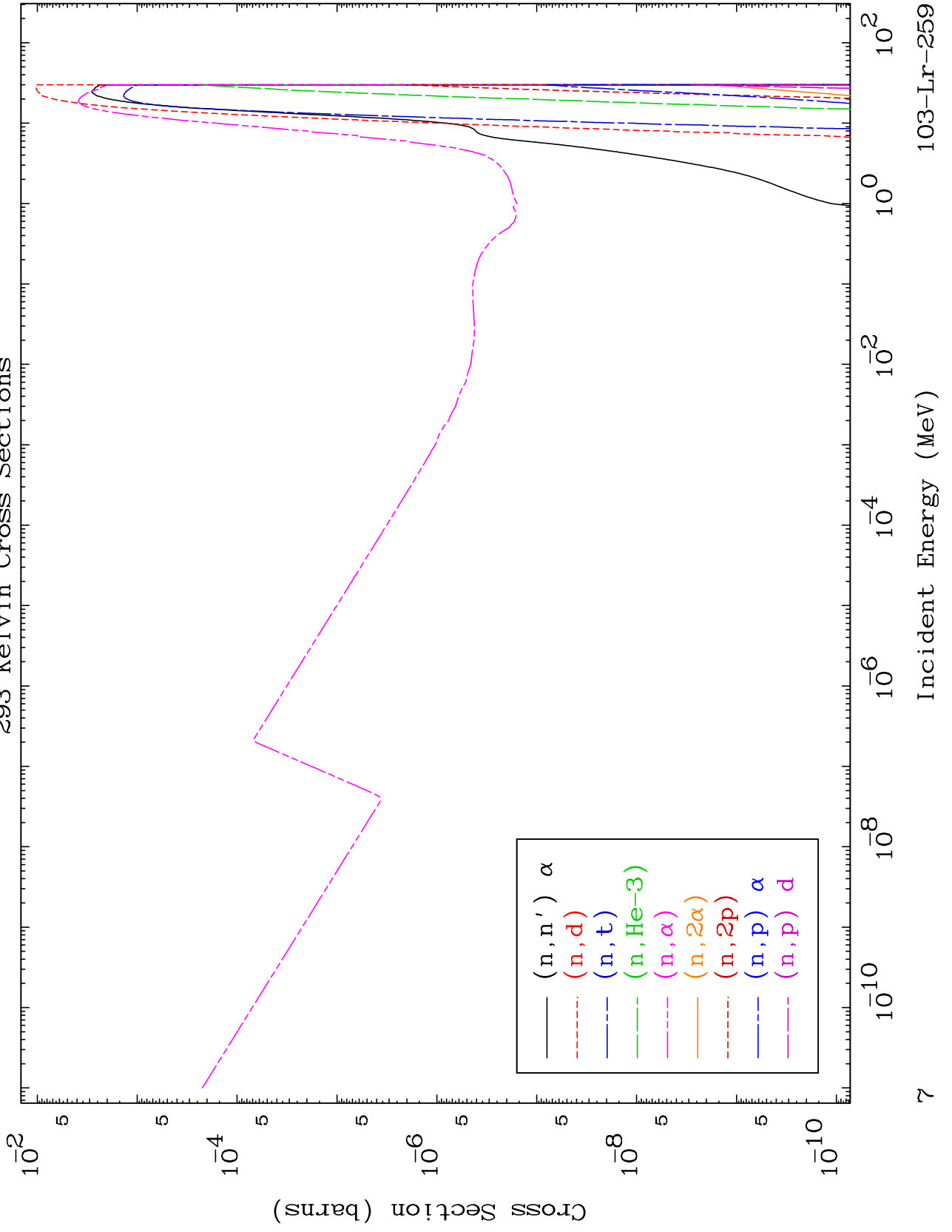
103-Lr-259



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Charged Particle
293 Kelvin Cross Sections

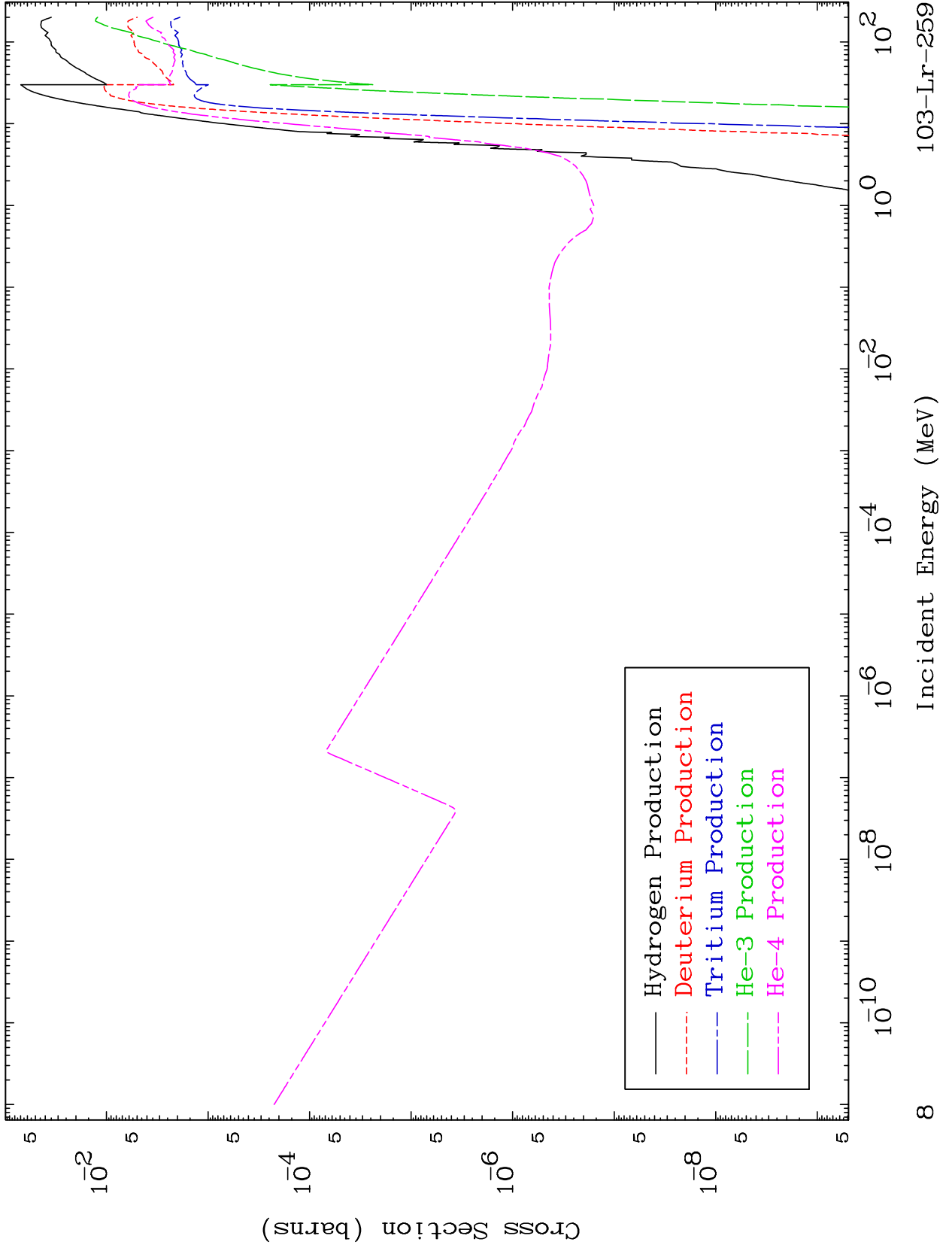
103-Lr-259



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Particle Production
293 Kelvin Cross Sections

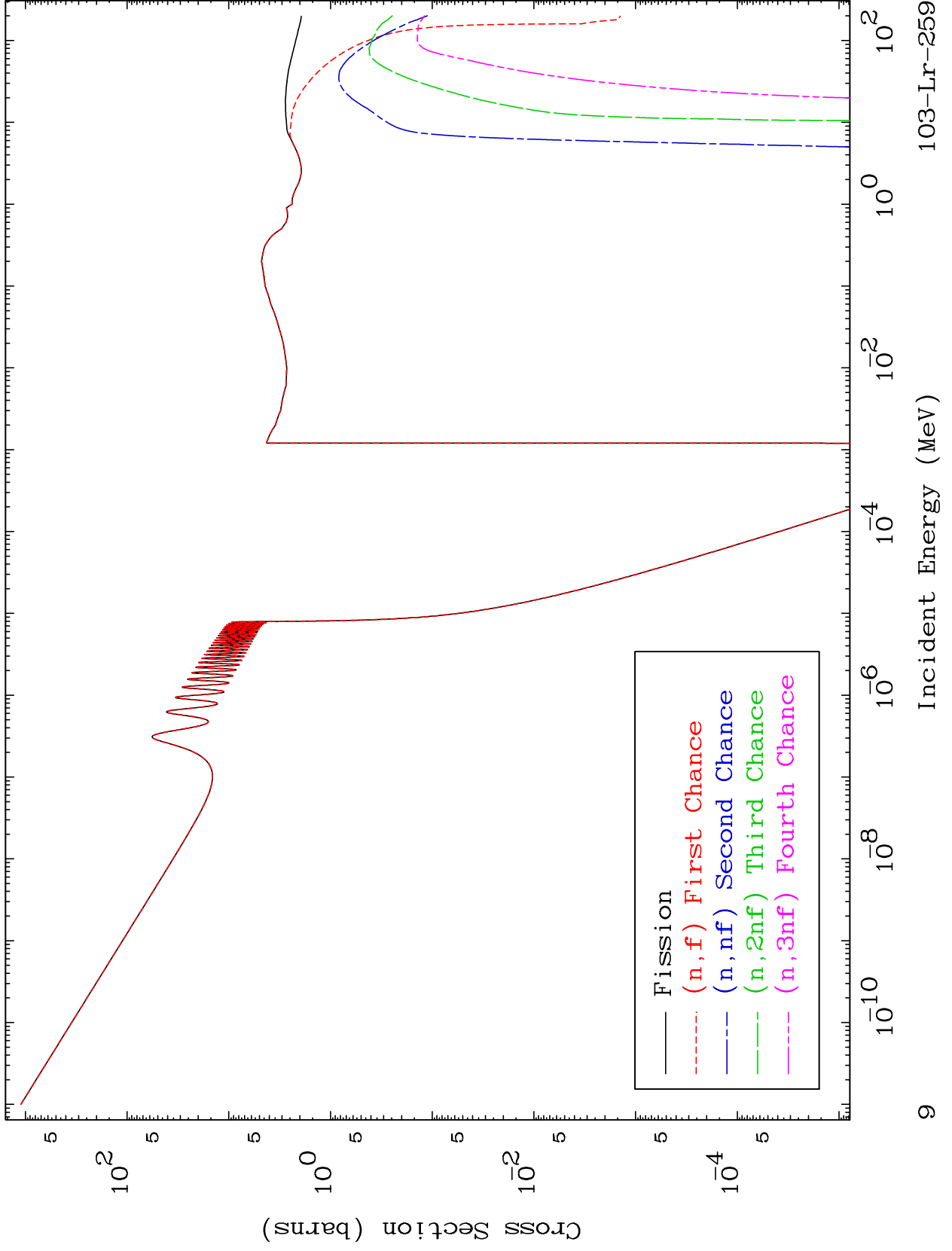
103-Lr-259



MAT 9988

Fission
293 Kelvin Cross Sections

103-Lr-259

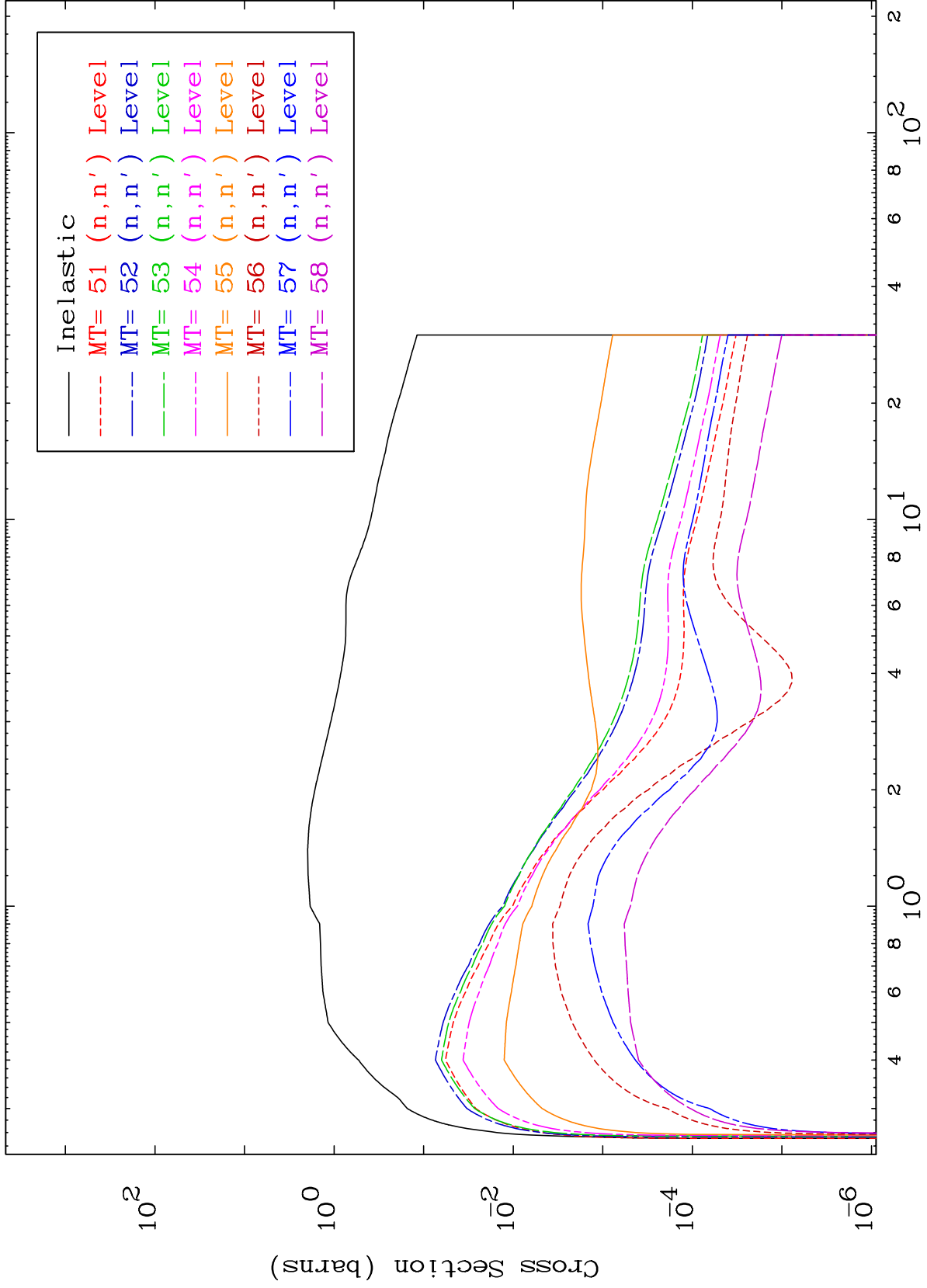


MAT 9988

(n,n') Levels

103-Lr-259

293 Kelvin Cross Sections



10

Incident Energy (MeV)

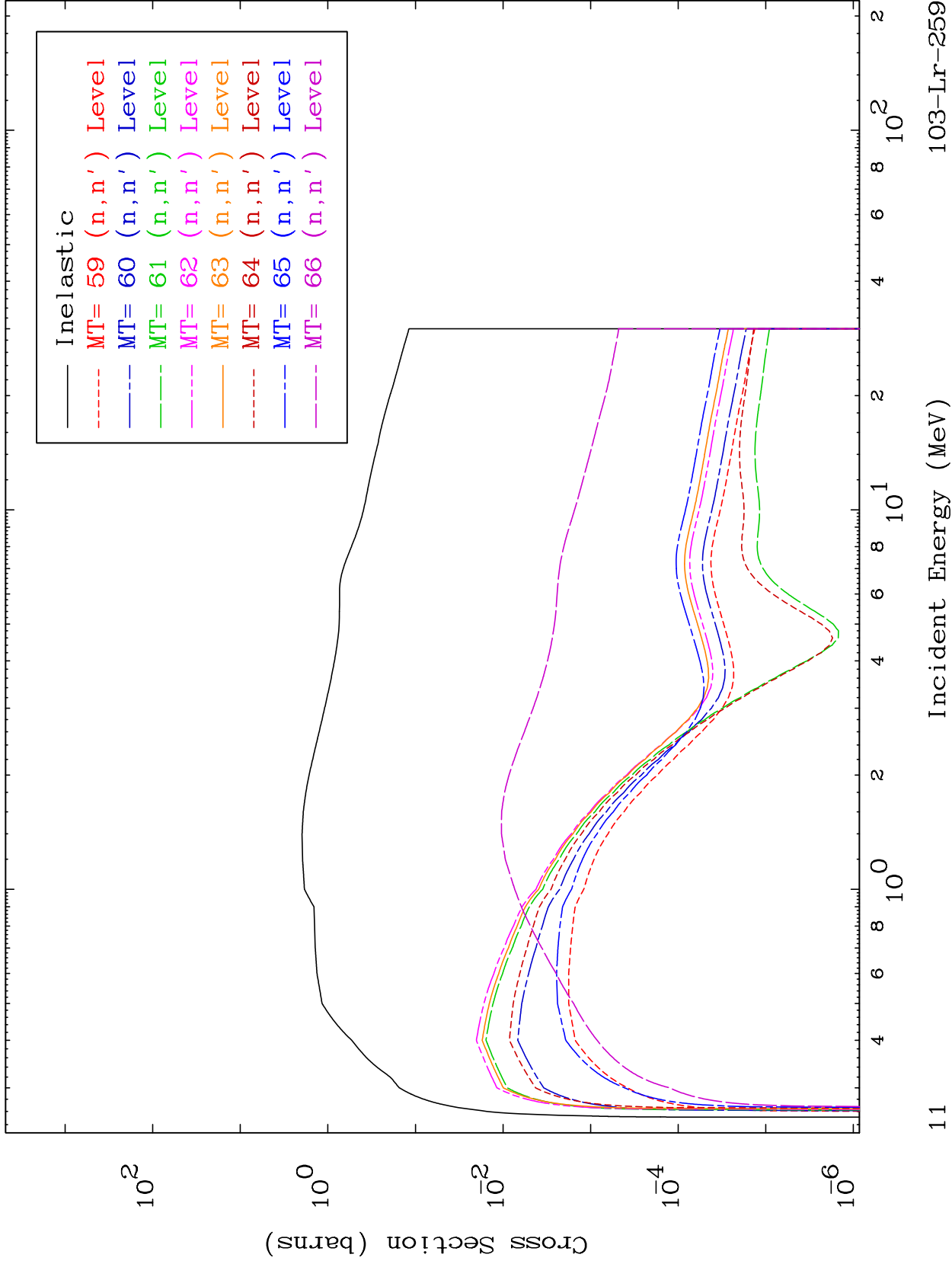
103-Lr-259

MAT 9988

(n,n') Levels

103-Lr-259

293 Kelvin Cross Sections



11

Incident Energy (MeV)

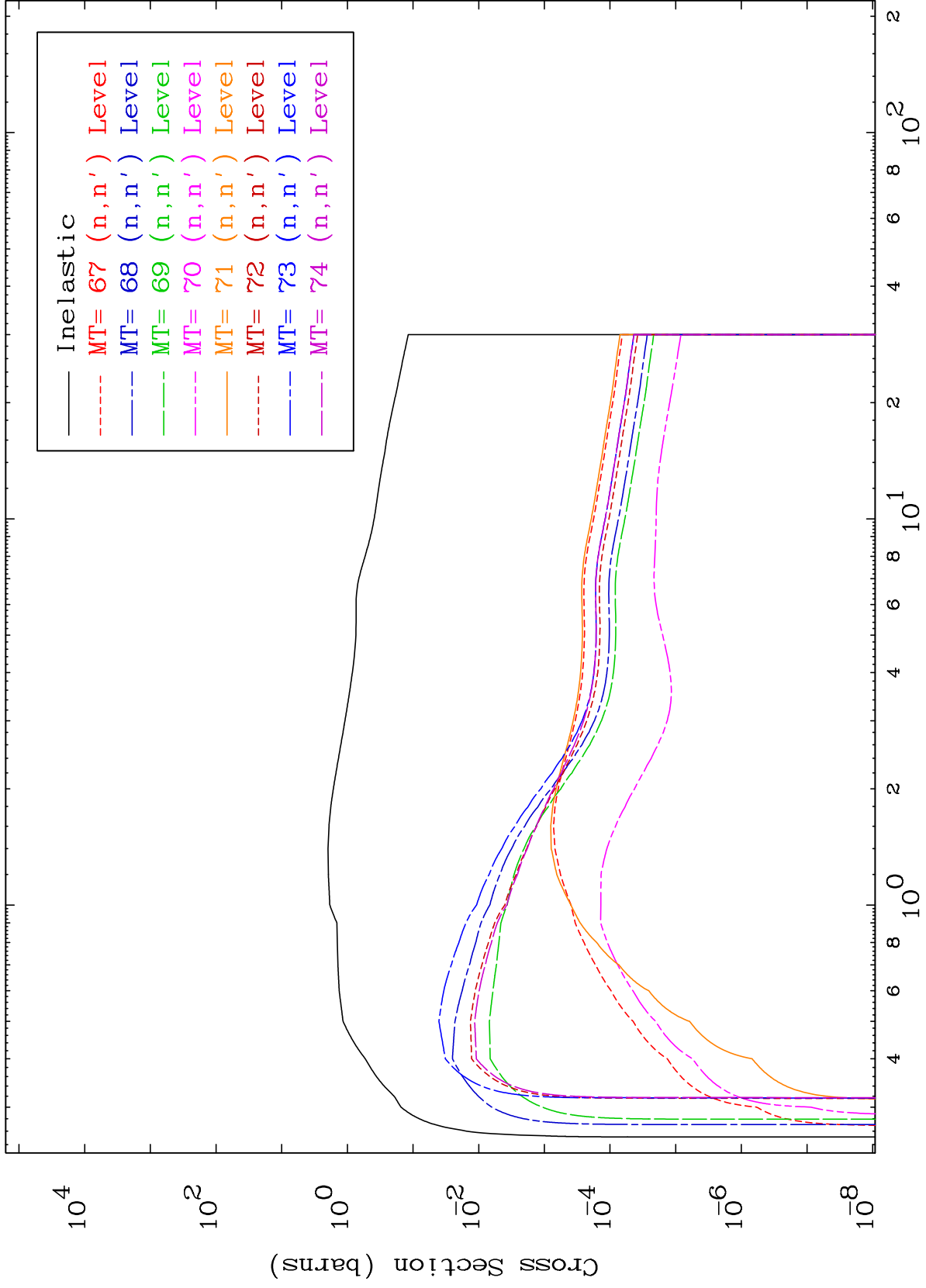
103-Lr-259

MAT 9988

(n,n') Levels

103-Lr-259

293 Kelvin Cross Sections

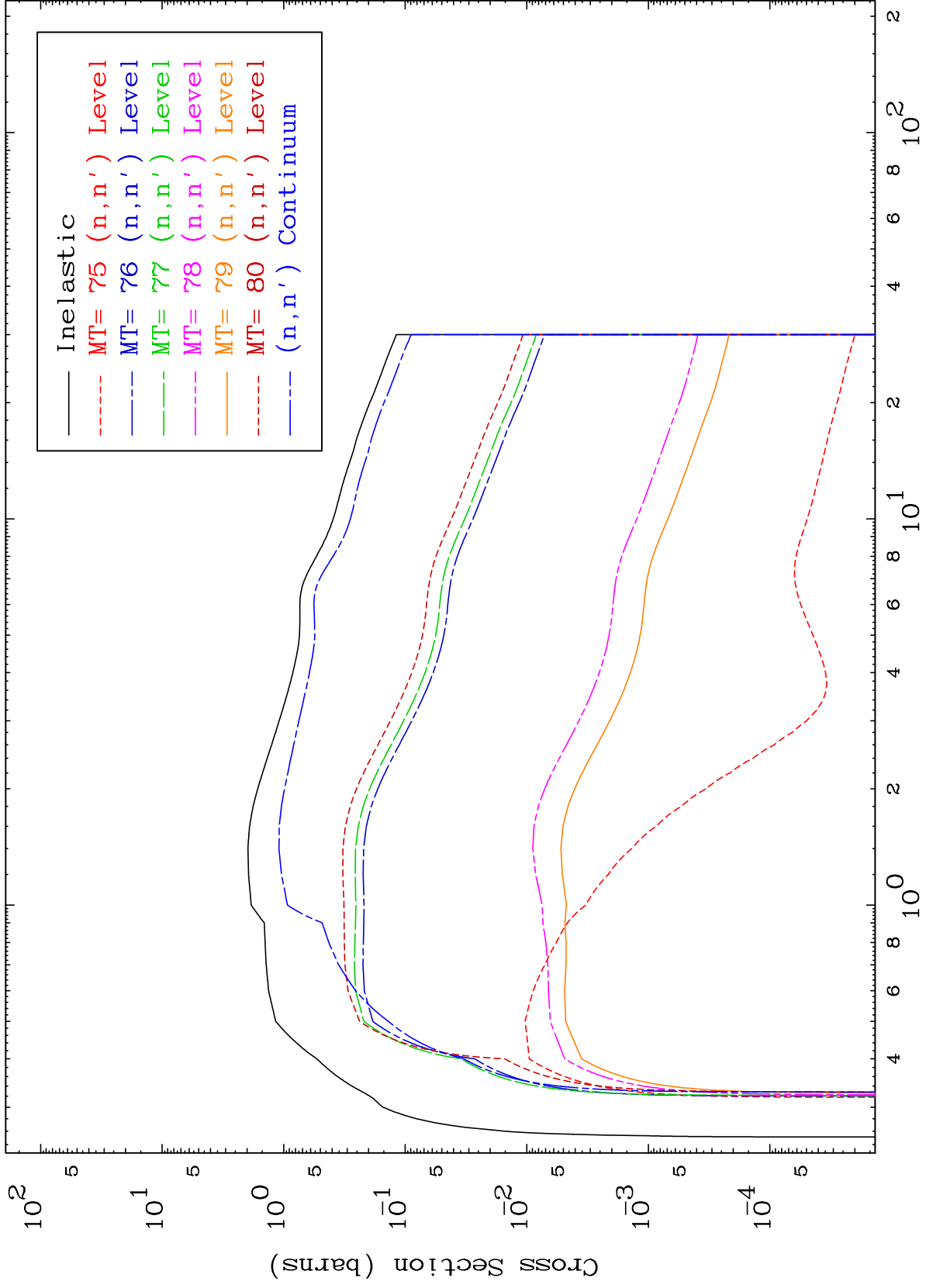


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Incident Energy (MeV)

103-Lr-259

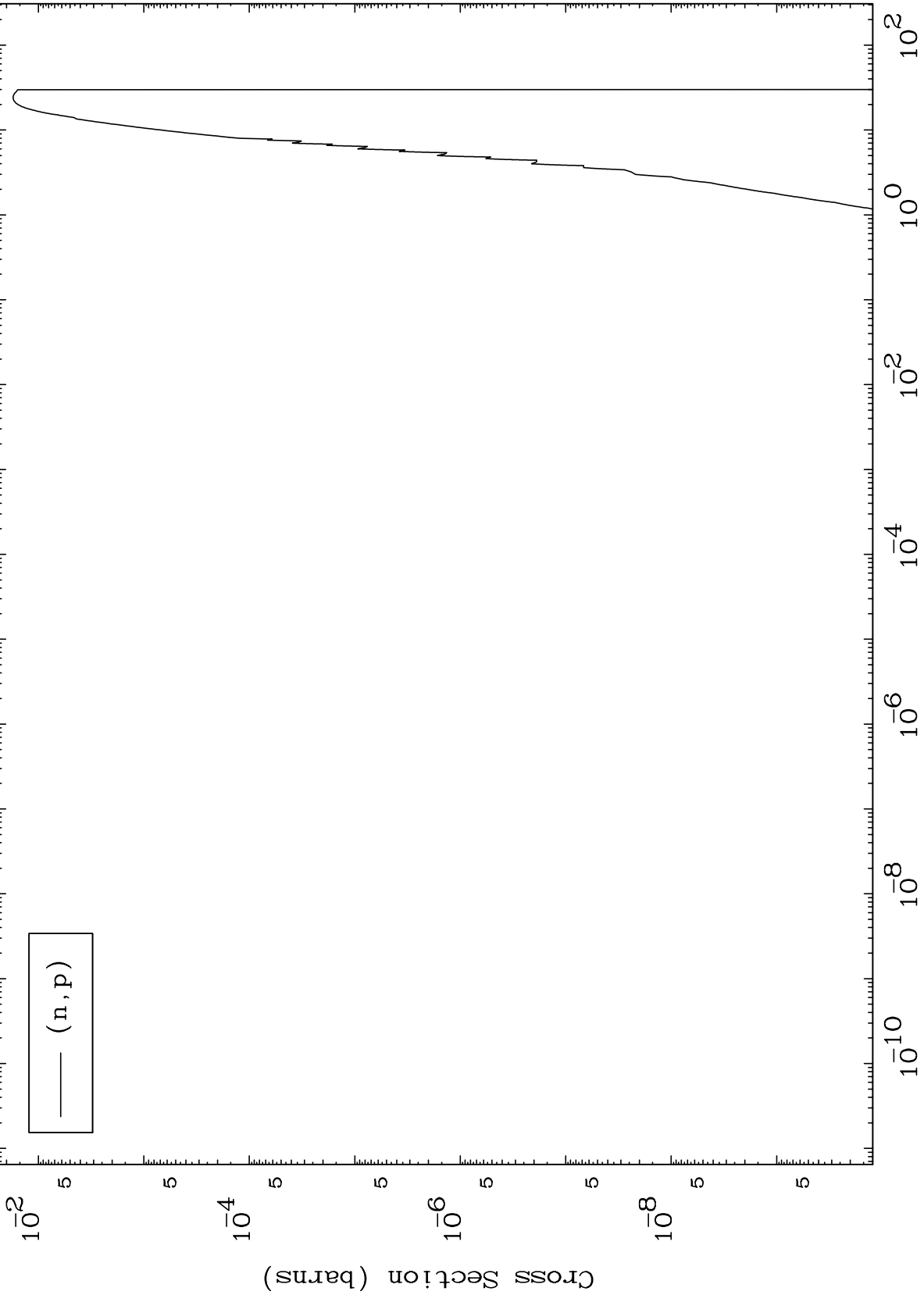
293 Kelvin Cross Sections



MAT 9988

(n,p) Levels
293 Kelvin Cross Sections

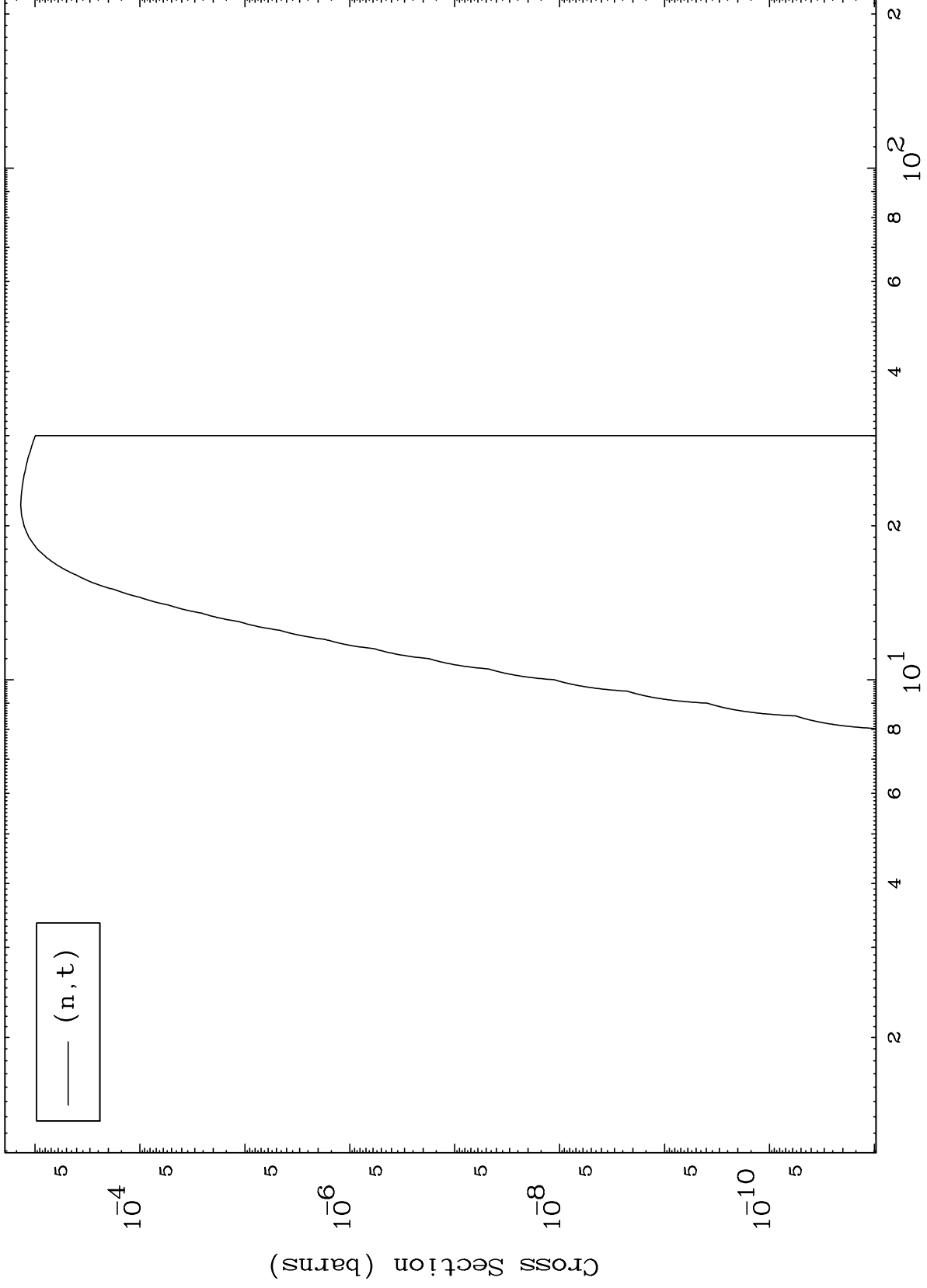
103-Lr-259



MAT 9988

(n,t) Levels
293 Kelvin Cross Sections

103-Lr-259



16

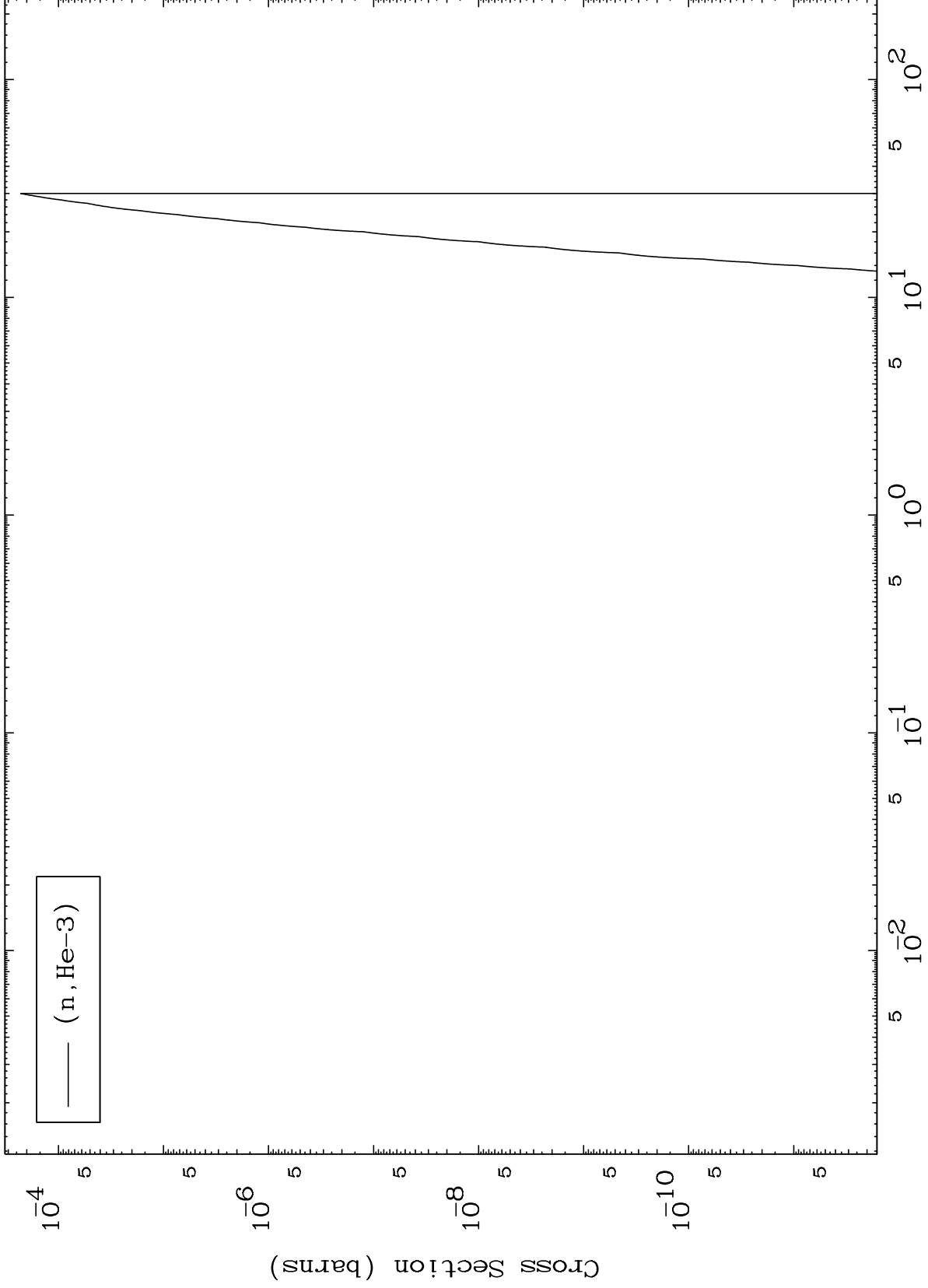
Incident Energy (MeV)

103-Lr-259

MAT 9988

(n,He3) Levels
293 Kelvin Cross Sections

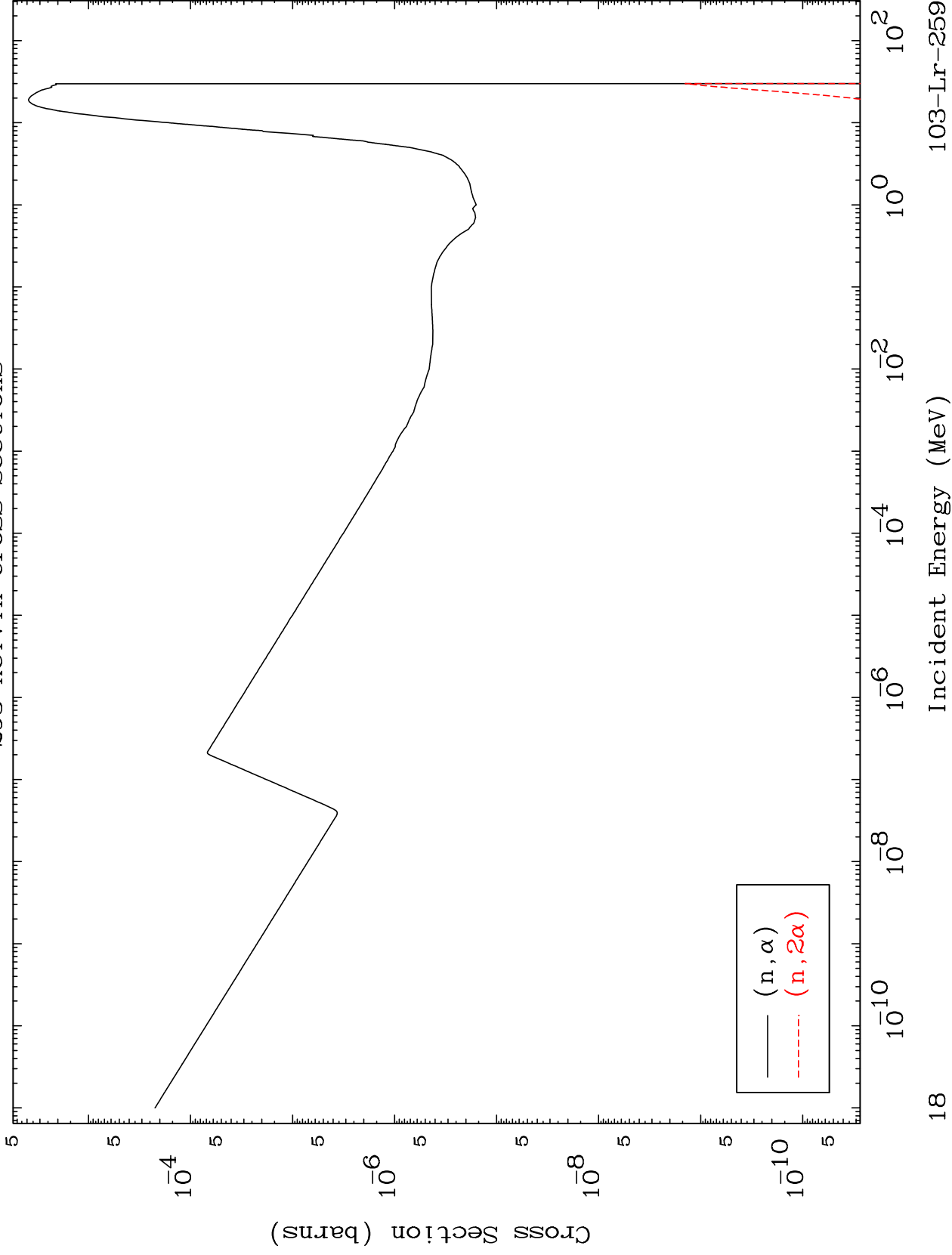
103-Lr-259



MAT 9988

(n, α) Levels
293 Kelvin Cross Sections

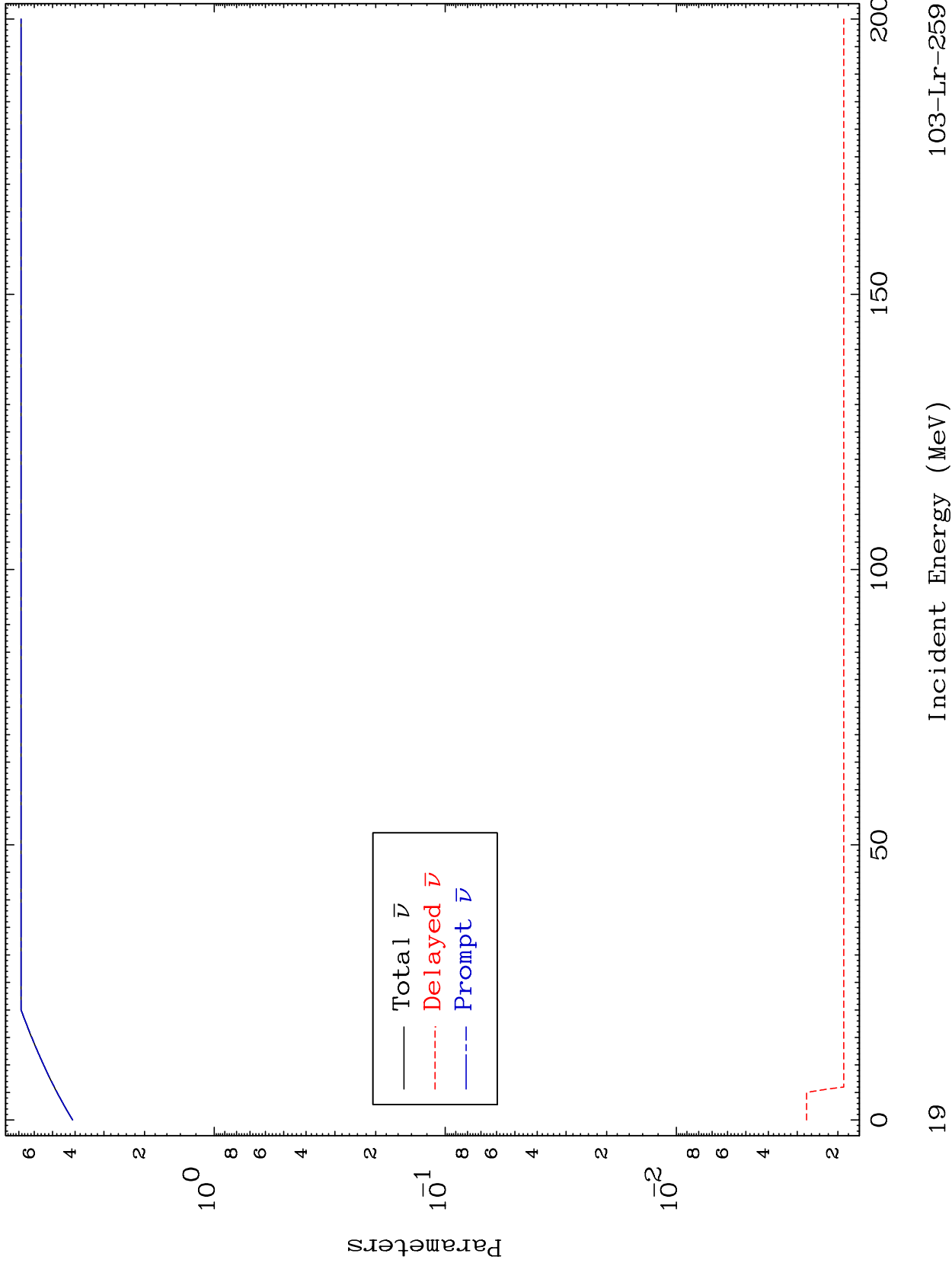
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Energy Release
Parameters

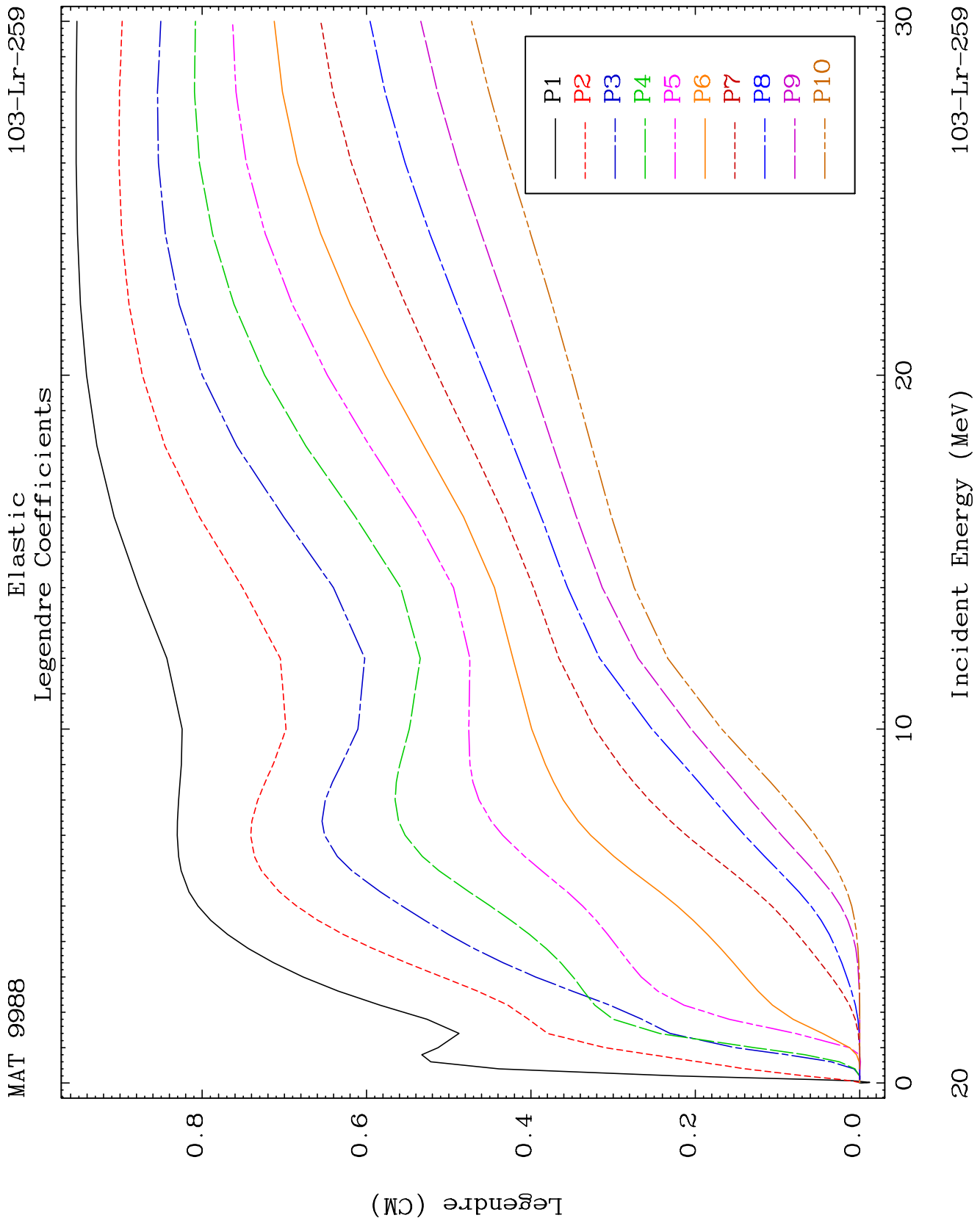
103-Lr-259



19

Incident Energy (MeV)

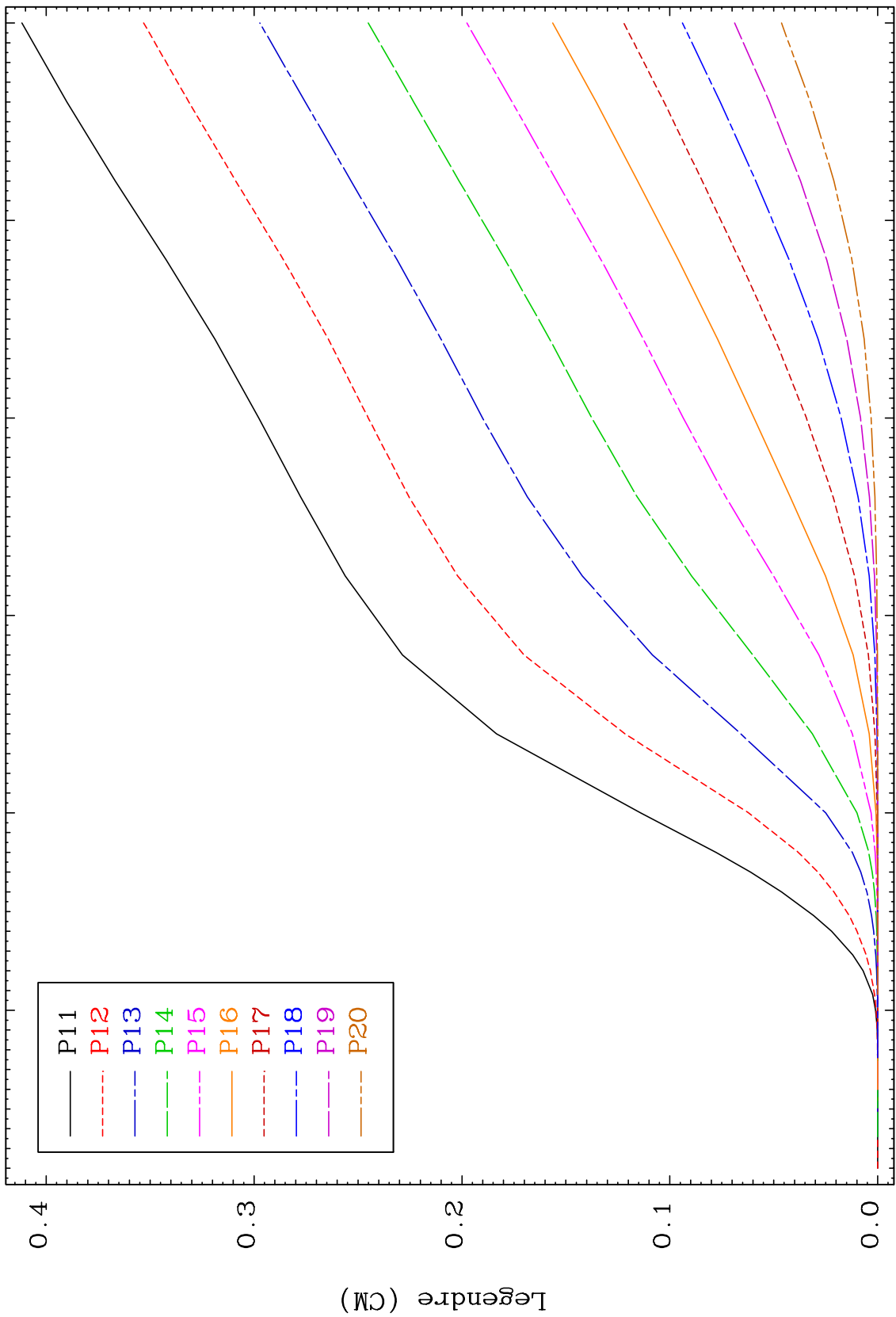
103-Lr-259



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Elastic Legendre Coefficients

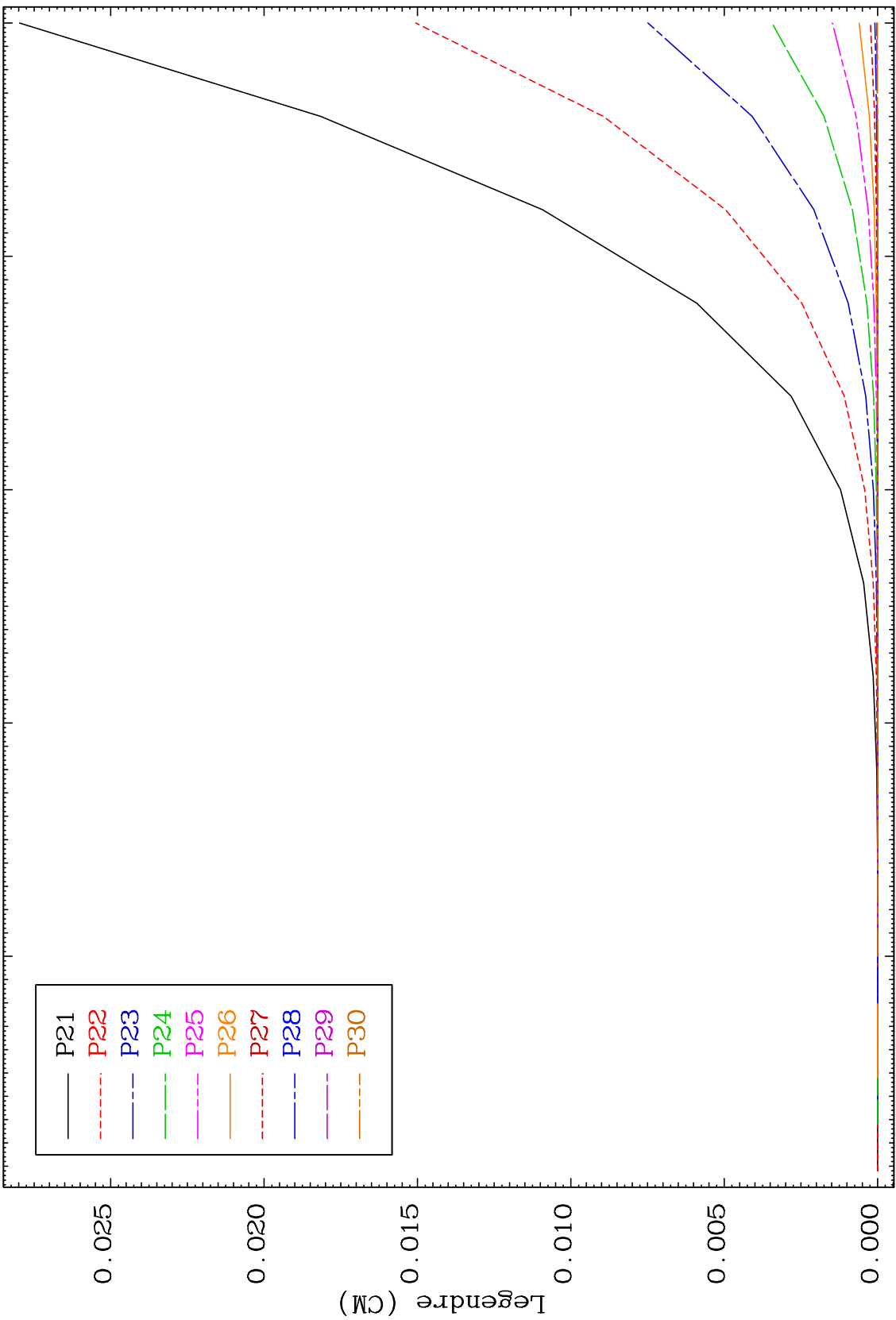
103-Lr-259



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Elastic Legendre Coefficients

103-Lr-259



22

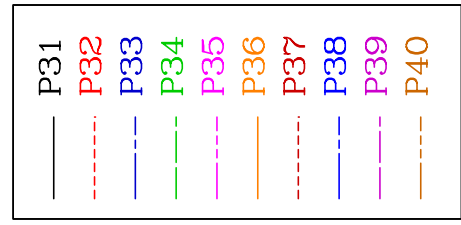
Incident Energy (MeV)

103-Lr-259

MAT 9988

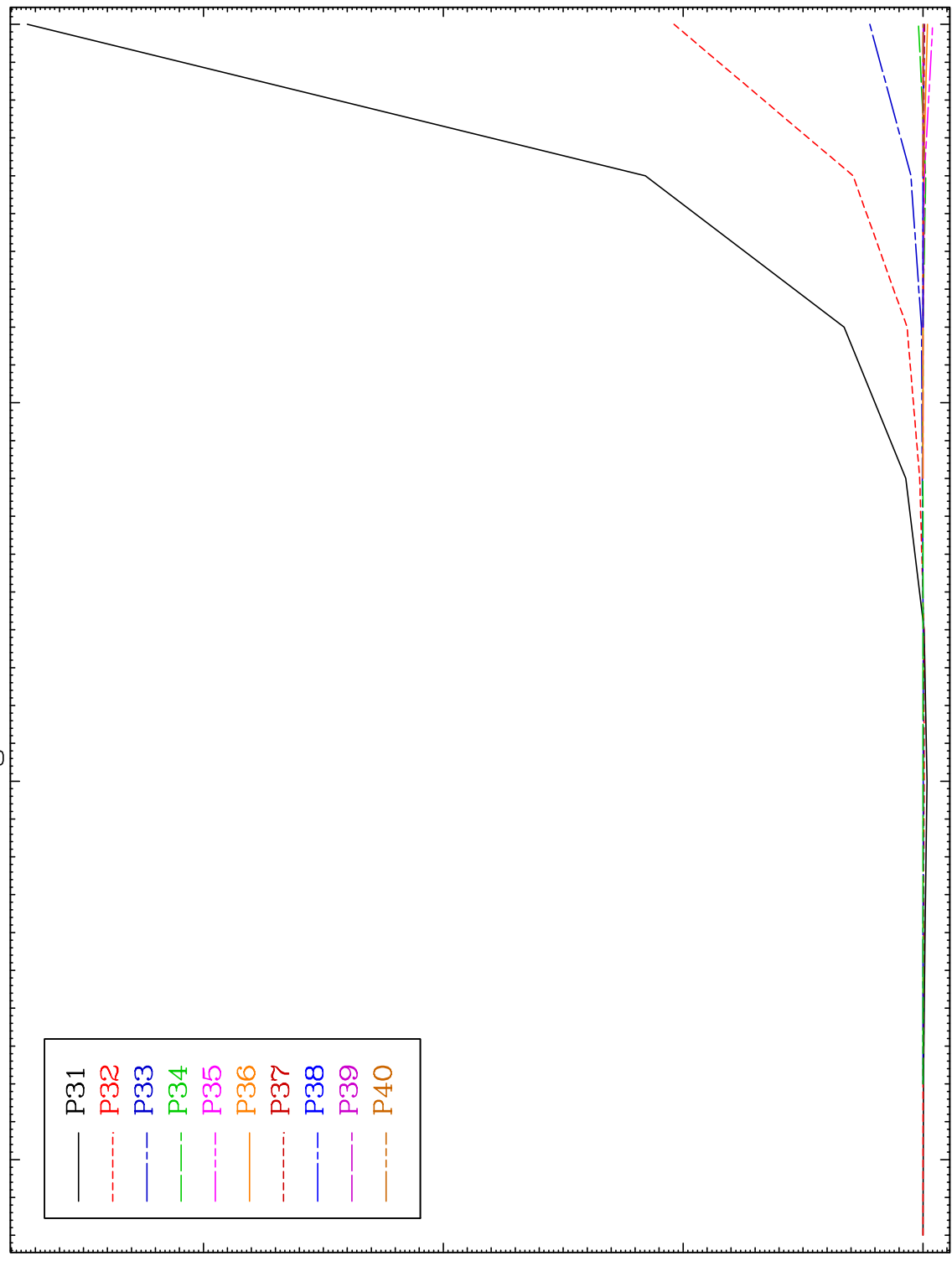
Elastic Legendre Coefficients

103-Lr-259



$\times 10^{-6}$

Legendre (CM)



30

25

20

15

23

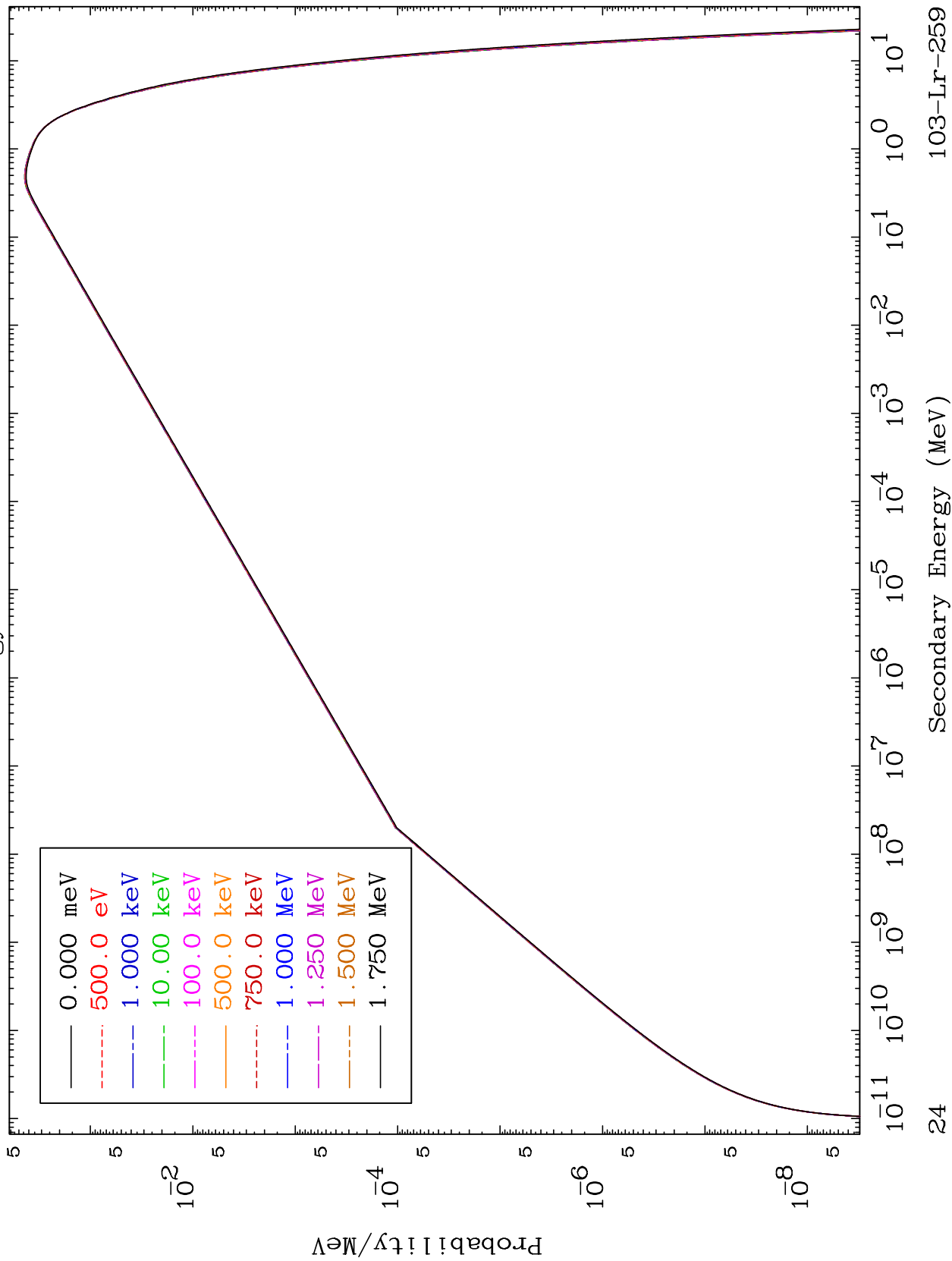
Incident Energy (MeV)

103-Lr-259

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Fission Energy Distributions

103-Lr-259



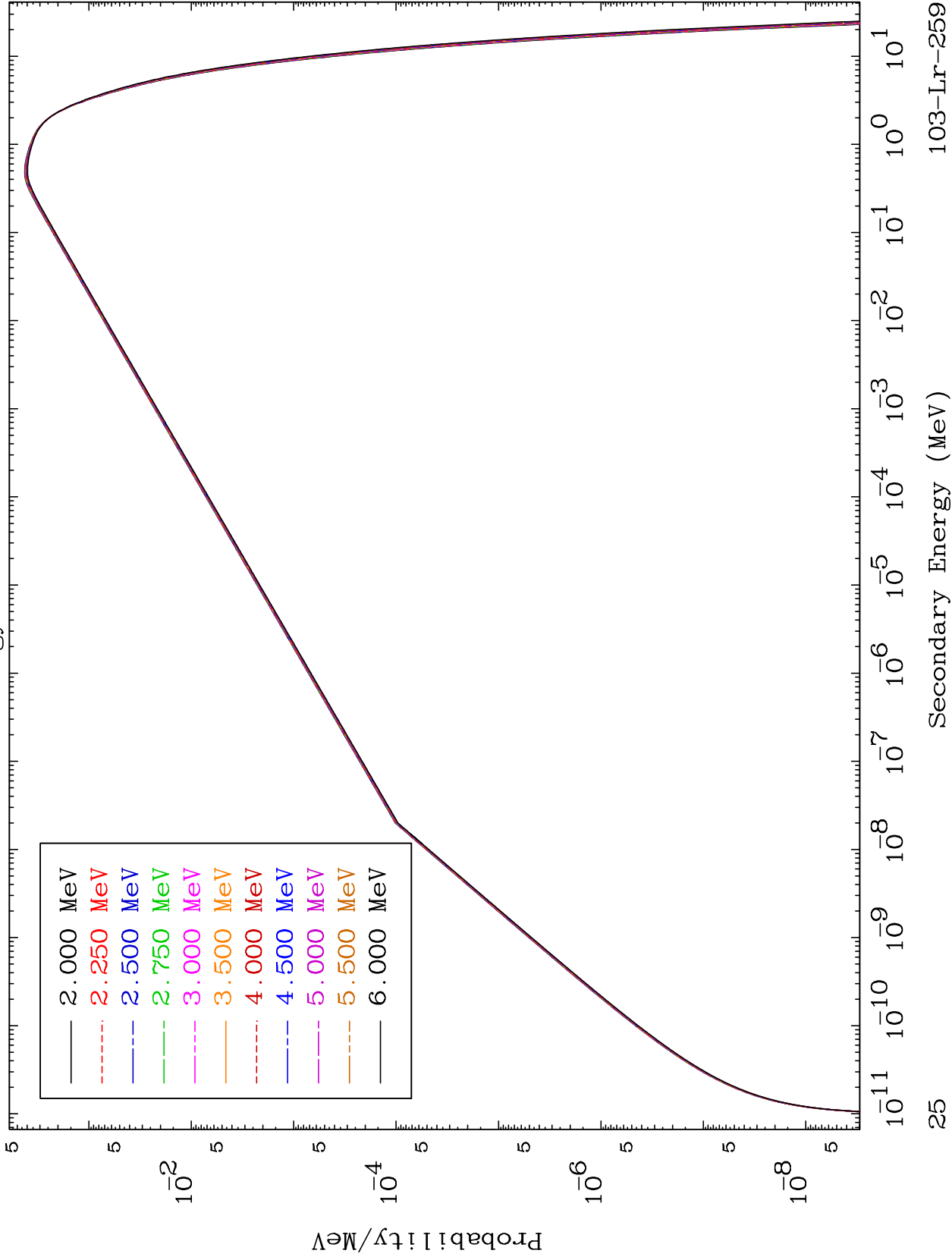
103-Lr-259

Secondary Energy (MeV)

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Fission Energy Distributions

103-Lr-259

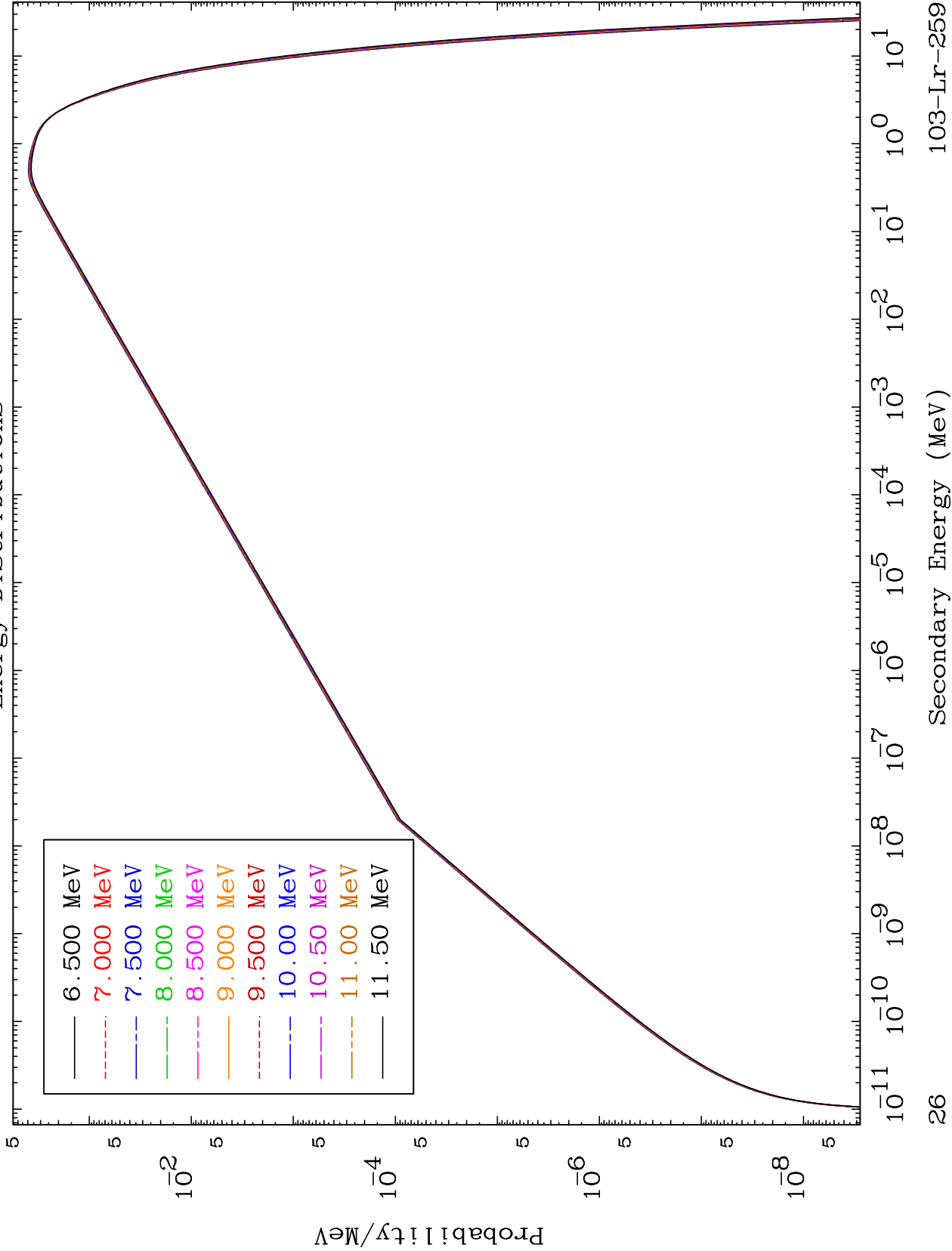


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Fission Energy Distributions

103-Lr-259



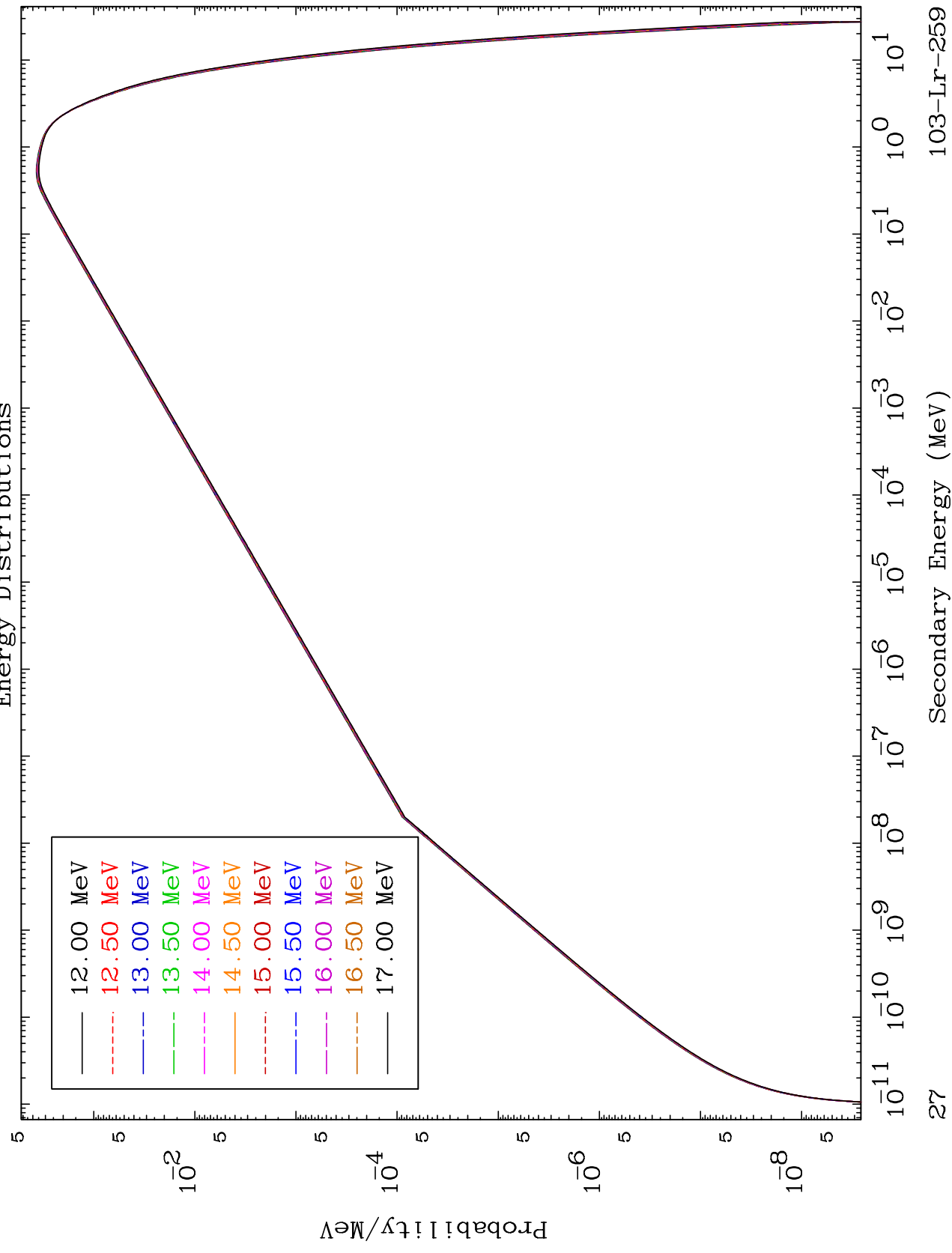
26

103-Lr-259

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Fission Energy Distributions

103-Lr-259



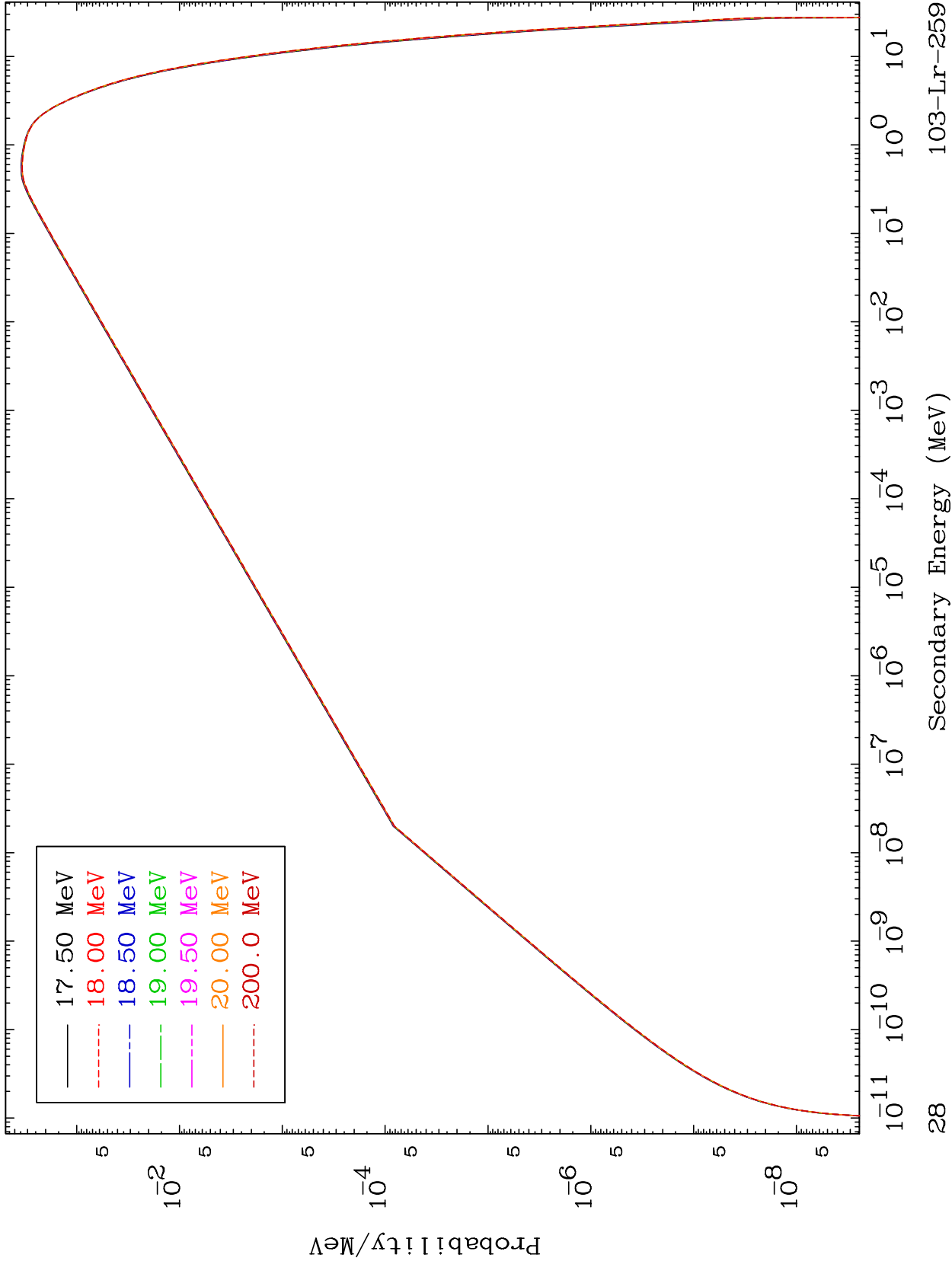
103-Lr-259

Secondary Energy (MeV)

MAT 9988

Fission Energy Distributions

103-Lr-259

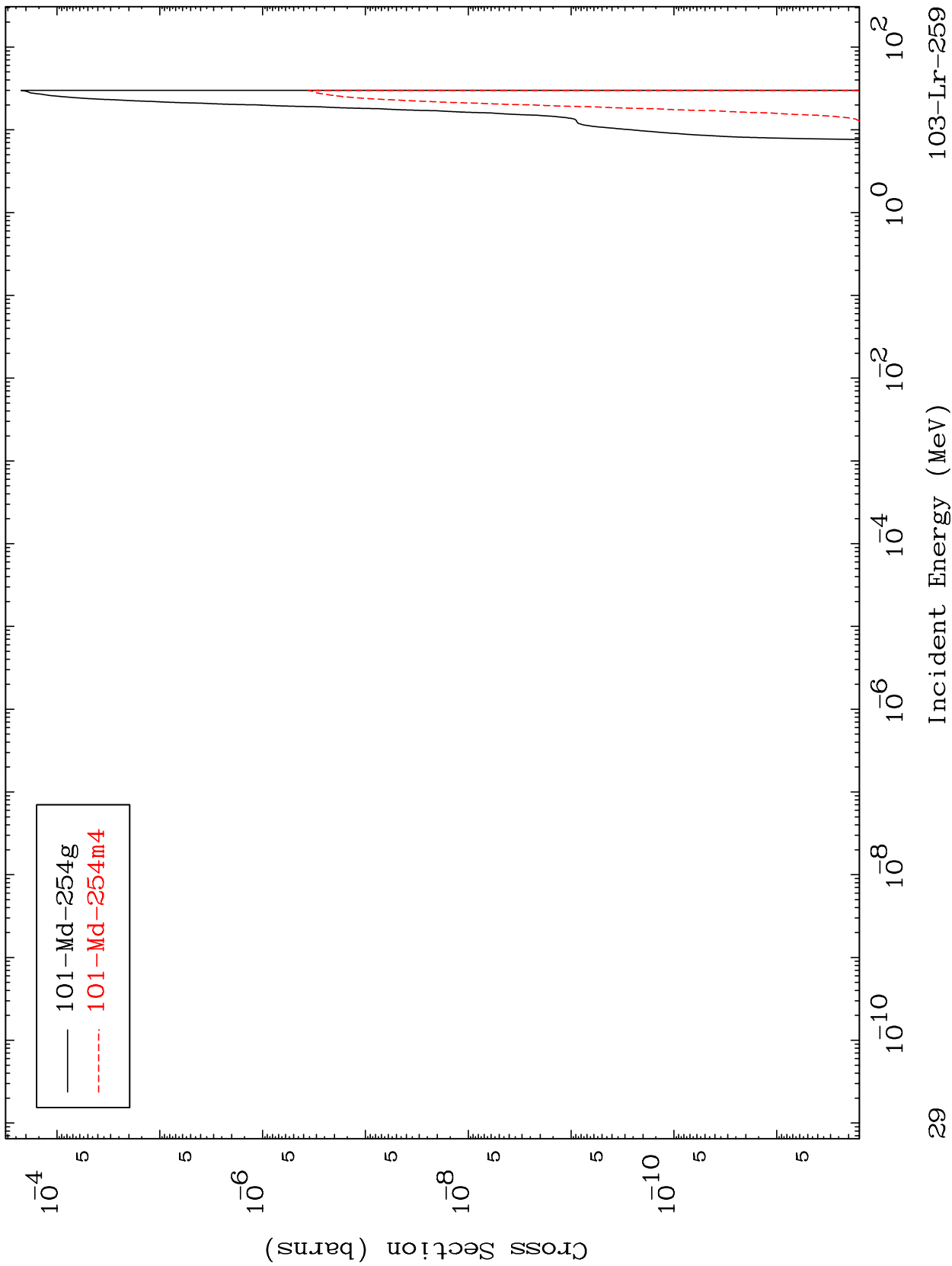


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(n,2n) α

103-Lr-259

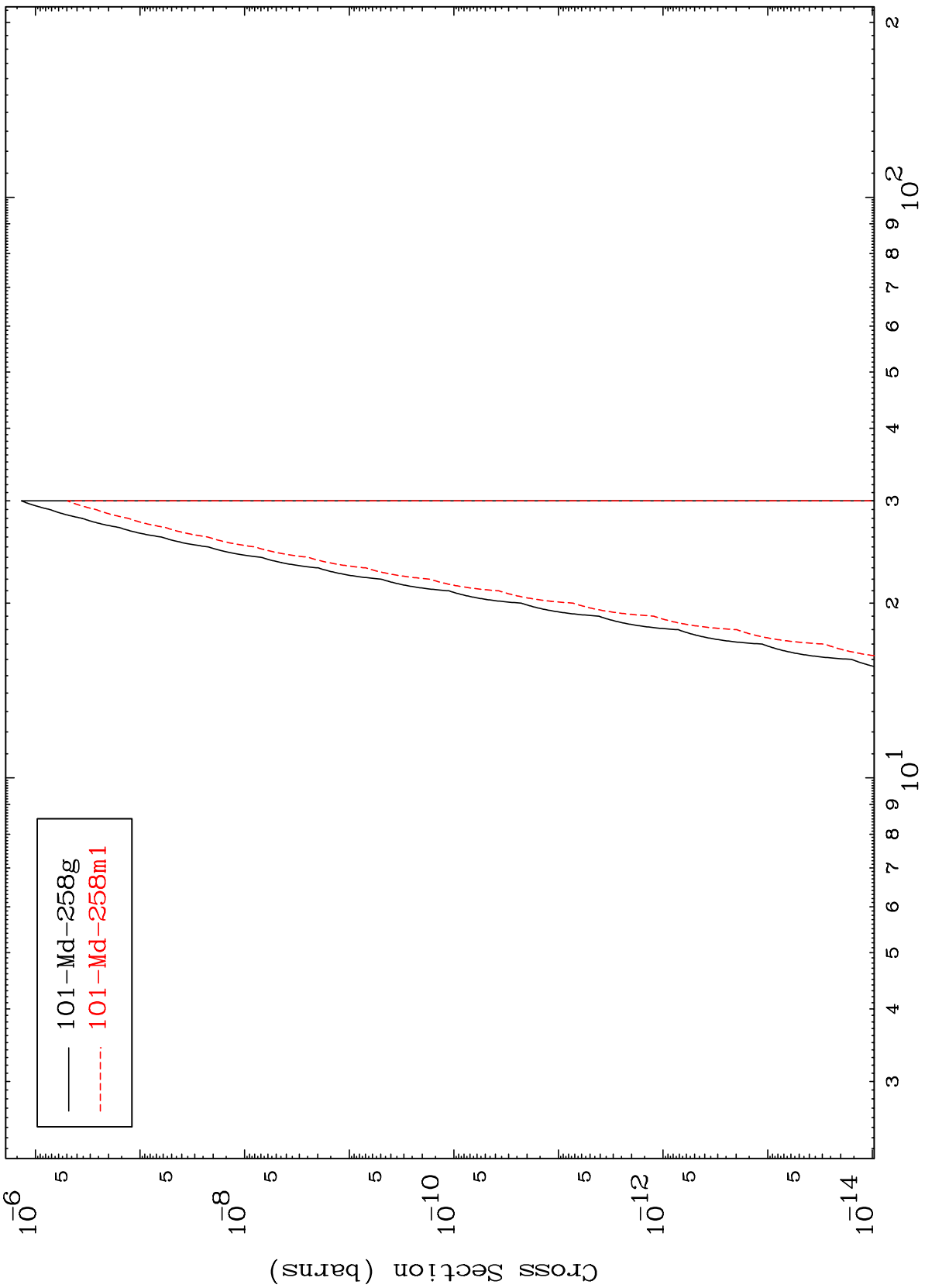
Radionuclide Production Cross Section



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103-Lr-259

(n,2p)
Radionuclide Production Cross Section



— 101-Md-258g
- - - 101-Md-258m1

Incident Energy (MeV)

103-Lr-259

30