

Program EVALPLOT  
(Version 2021-1)

by

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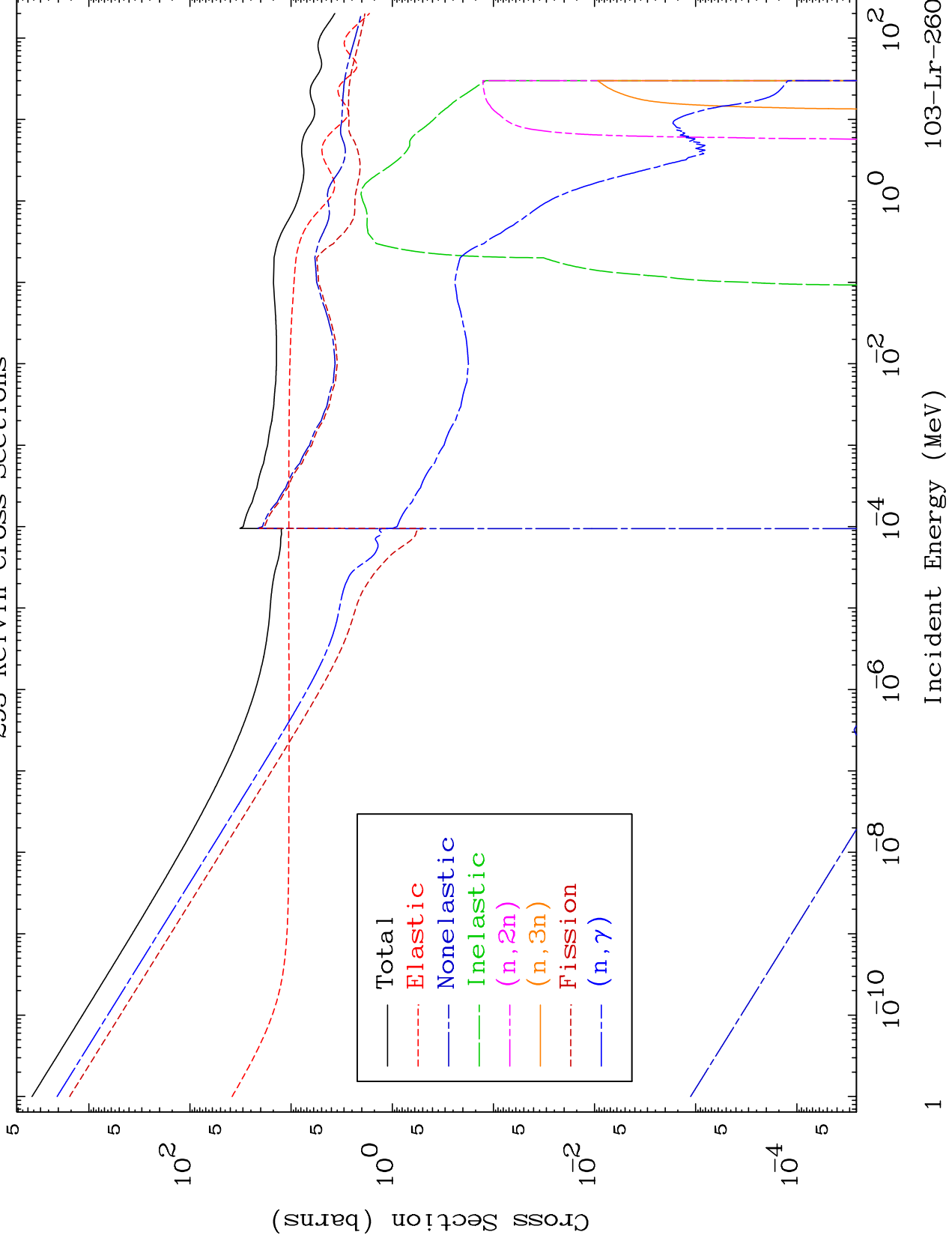
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9989

Neutron Major  
293 Kelvin Cross Sections

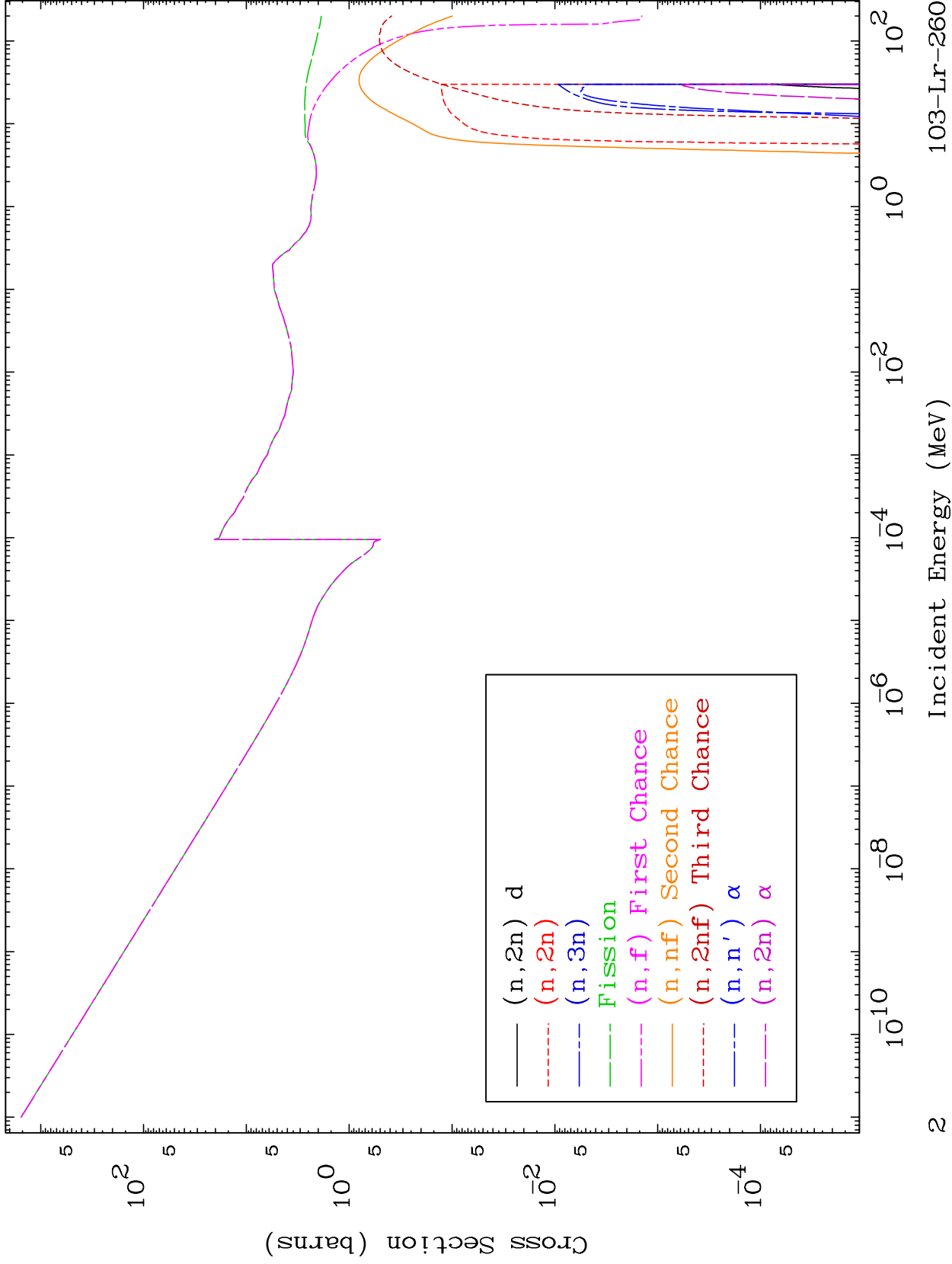
103-Lr-260



MAT 9989

Neutron Absorption  
293 Kelvin Cross Sections

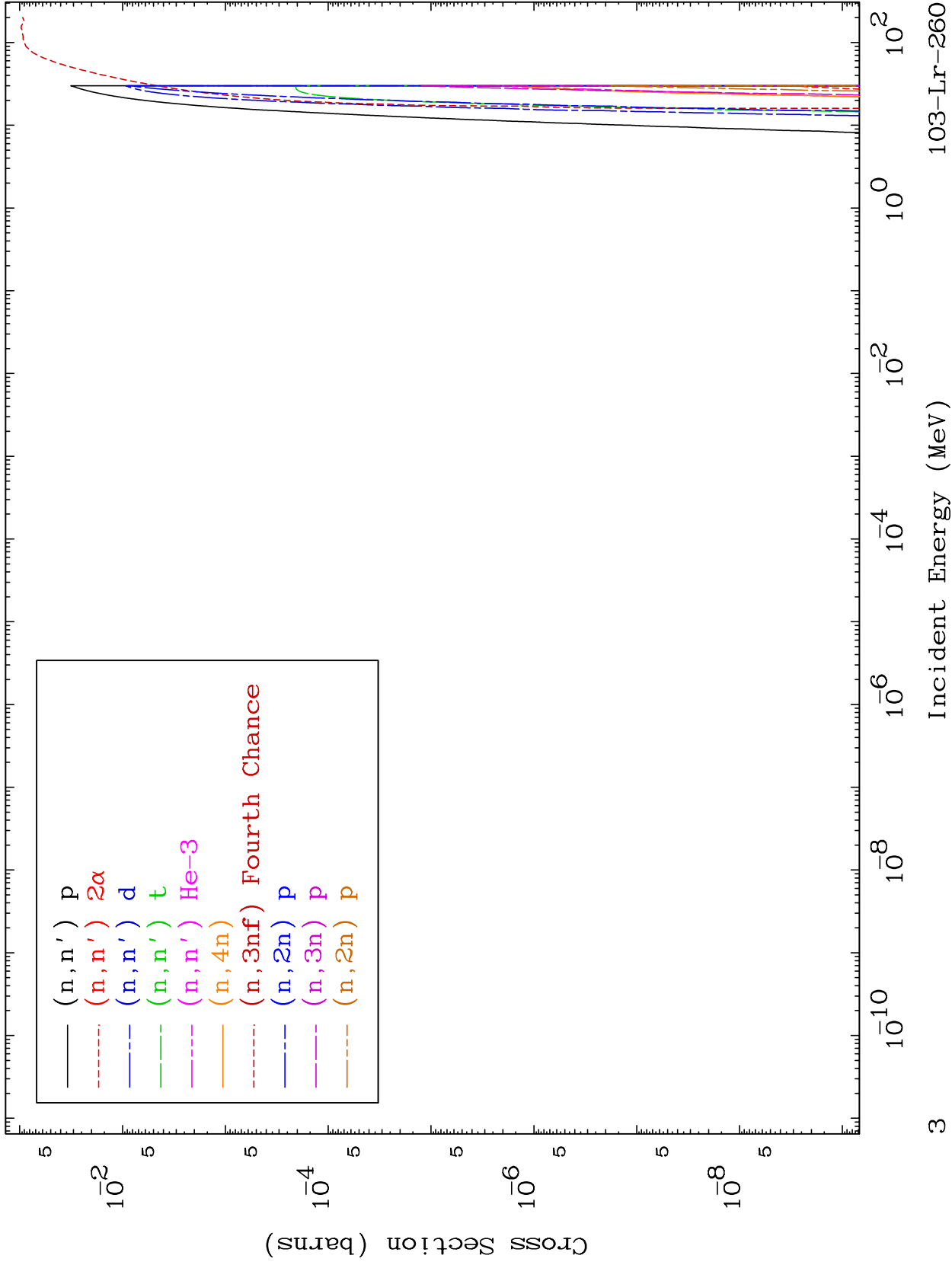
103-Lr-260



MAT 9989

Neutron Absorption  
293 Kelvin Cross Sections

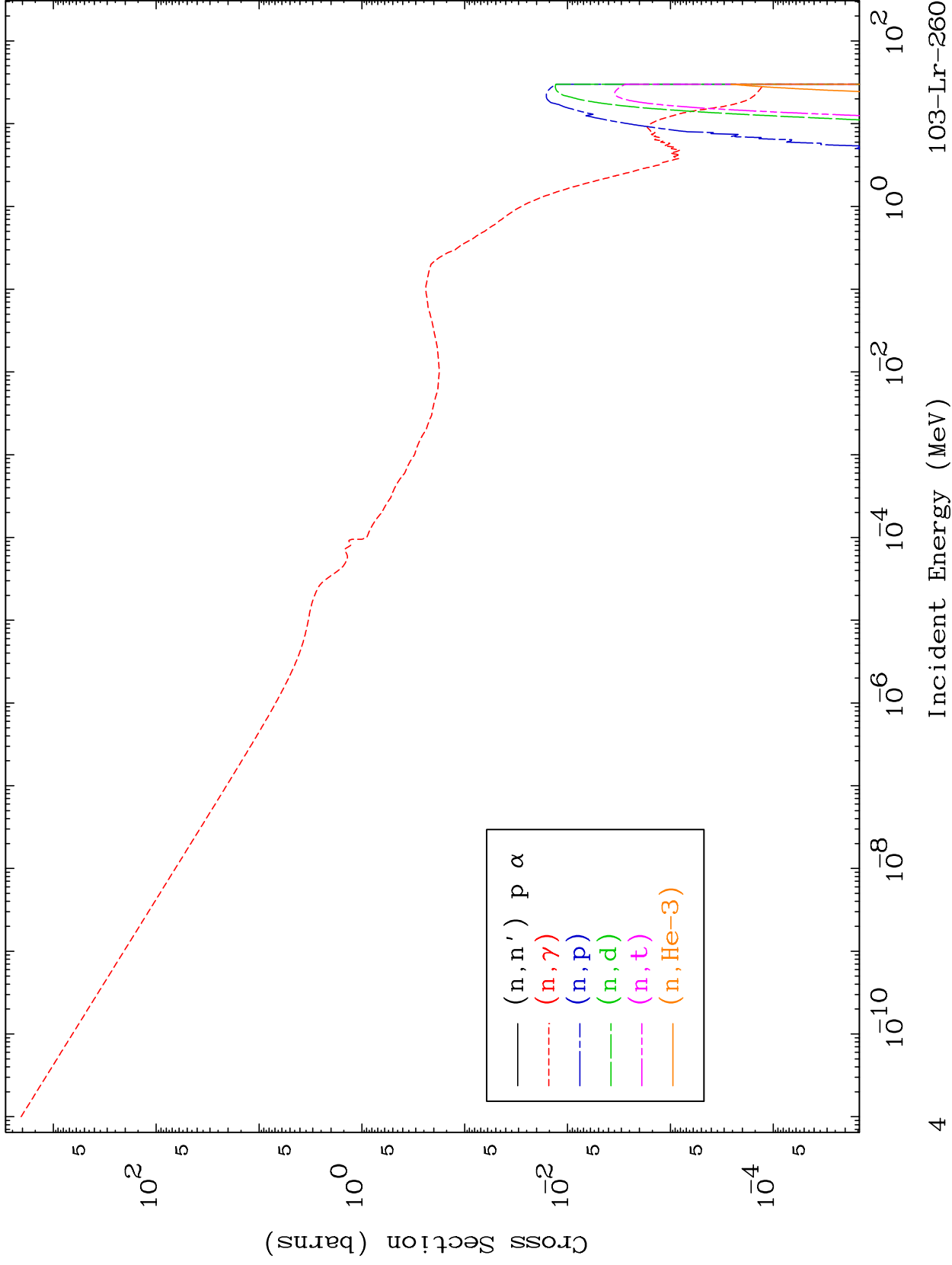
103-Lr-260

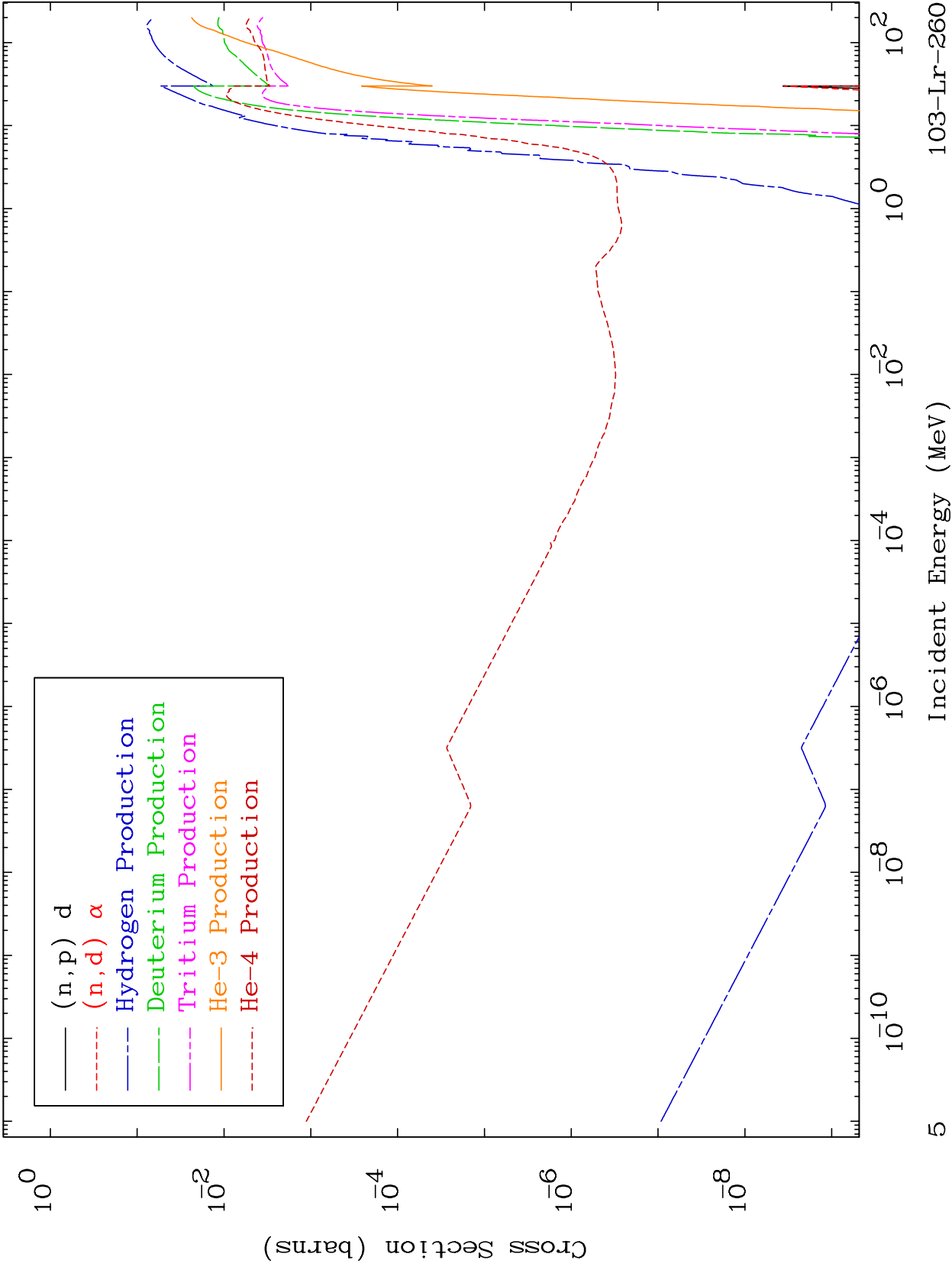


MAT 9989

Neutron Absorption  
293 Kelvin Cross Sections

103-Lr-260

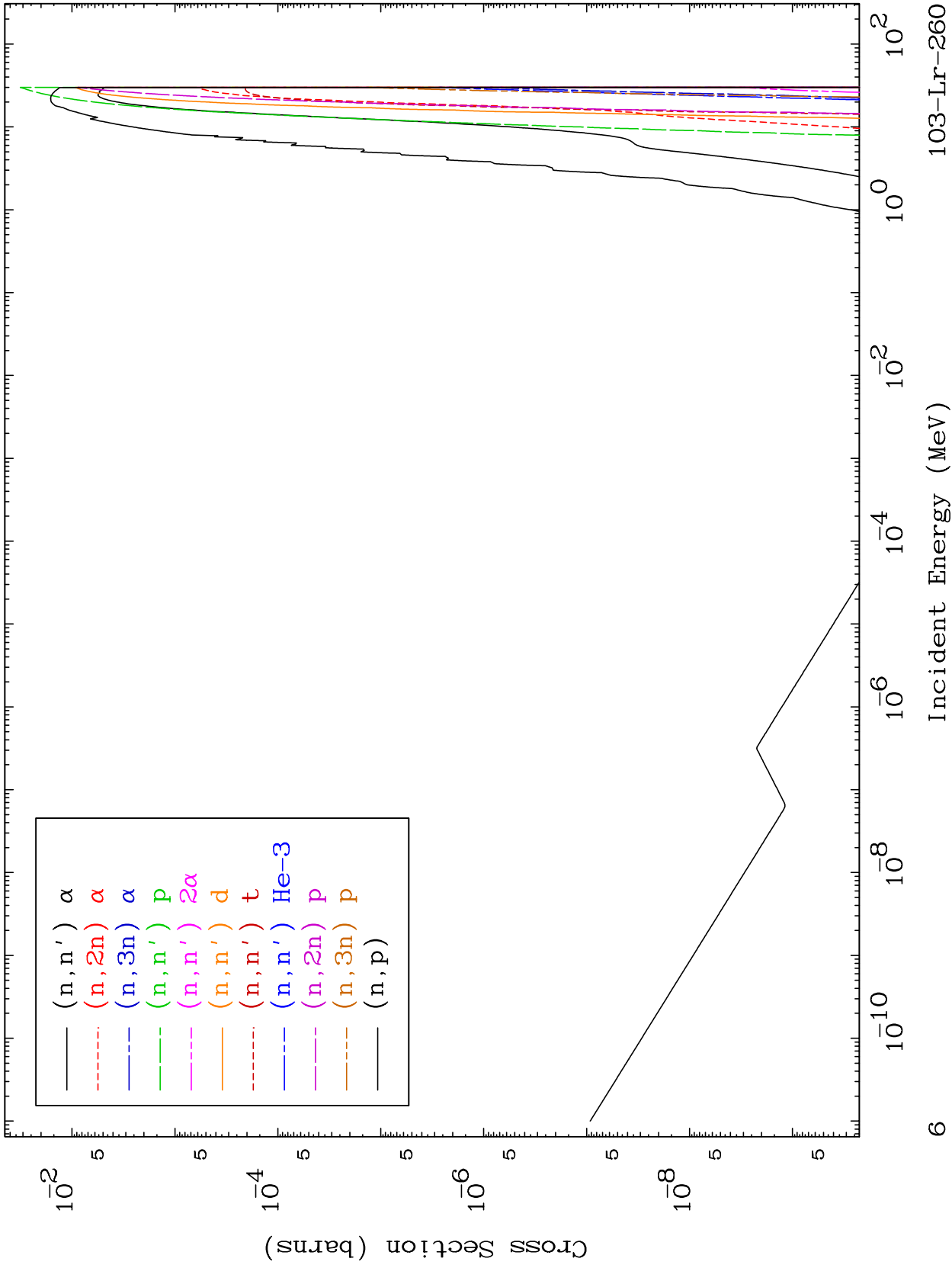




MAT 9989

Charged Particle  
293 Kelvin Cross Sections

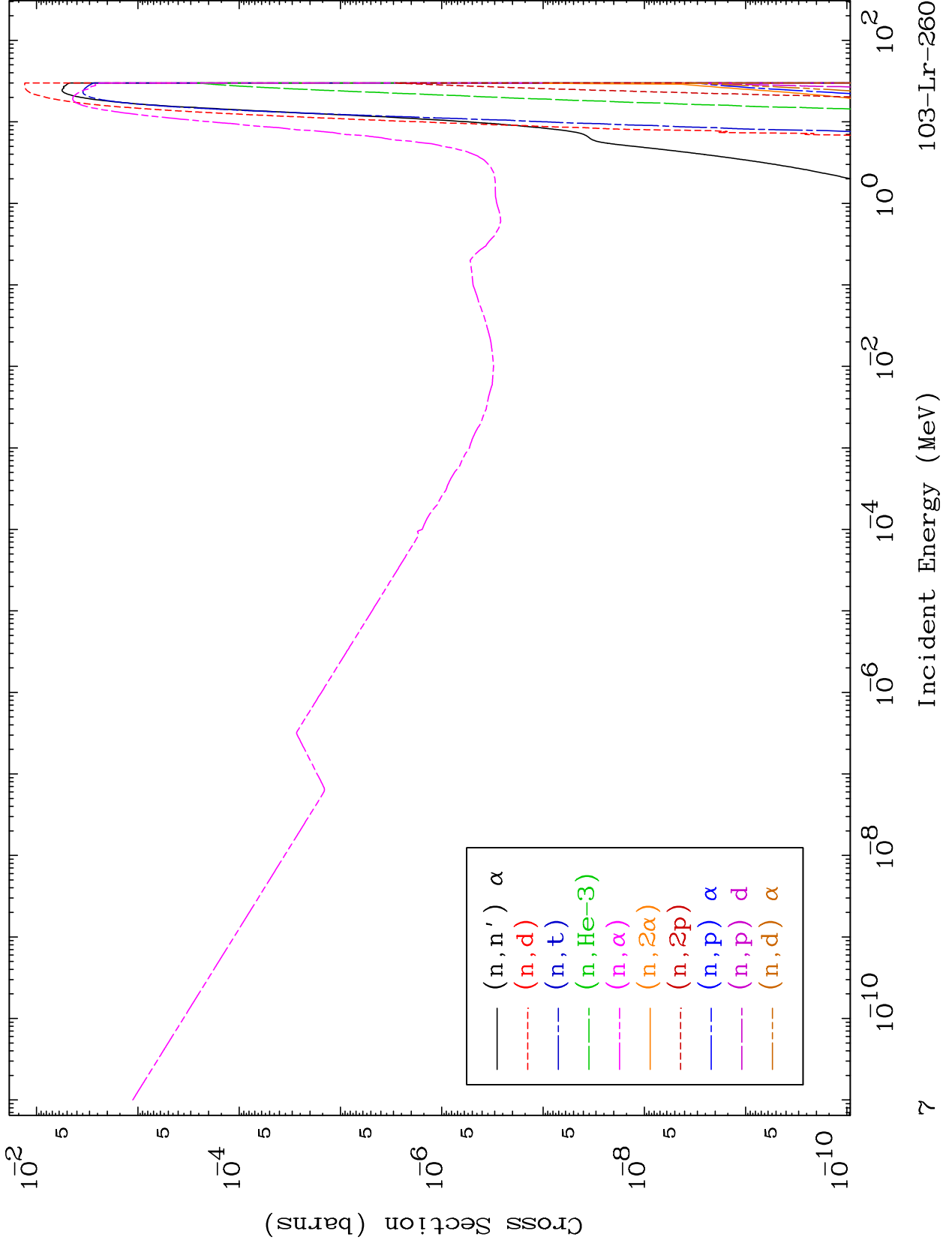
103-Lr-260



MAT 9989

Charged Particle  
293 Kelvin Cross Sections

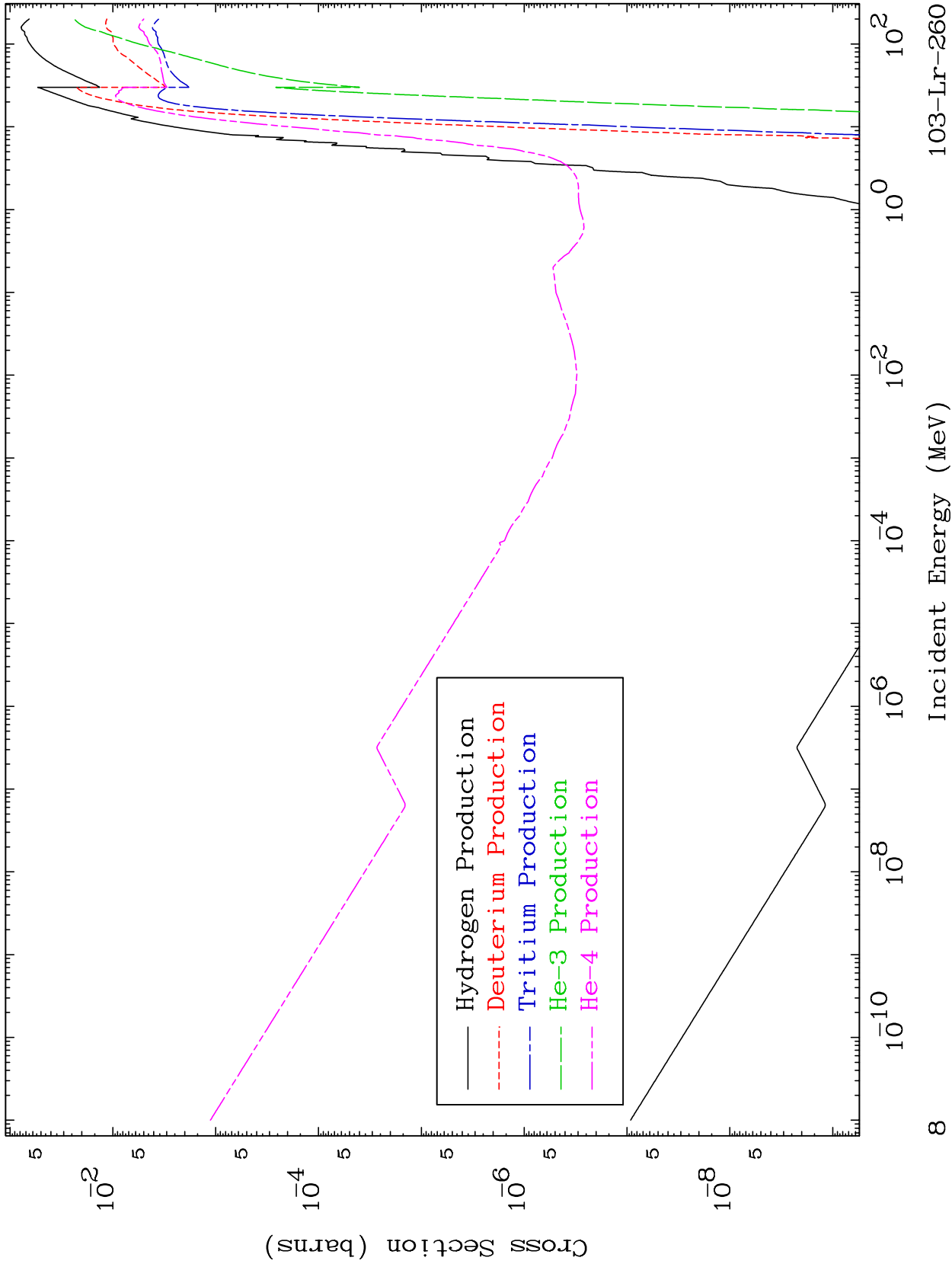
103-Lr-260



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Particle Production  
293 Kelvin Cross Sections

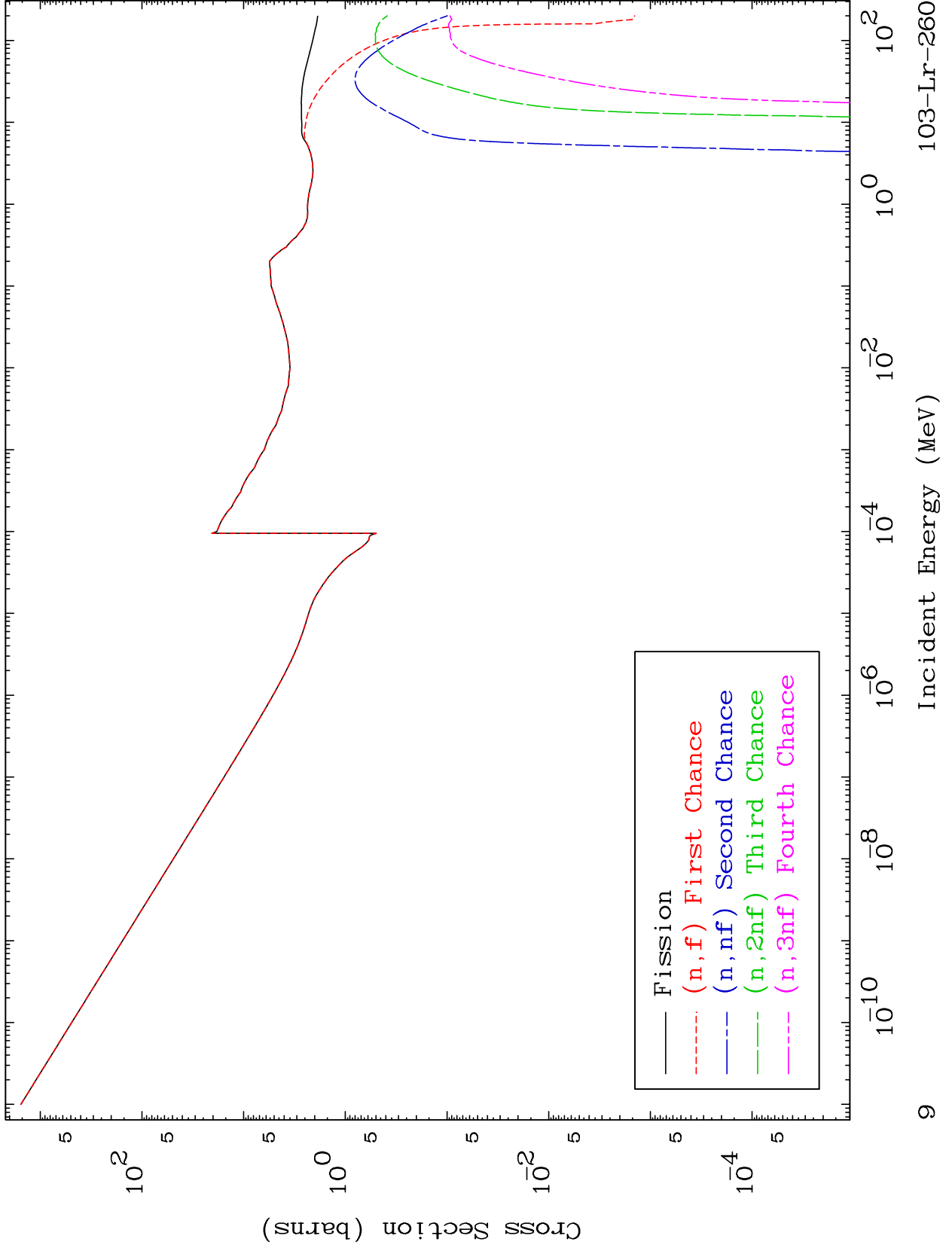
103-Lr-260



MAT 9989

Fission  
293 Kelvin Cross Sections

103-Lr-260

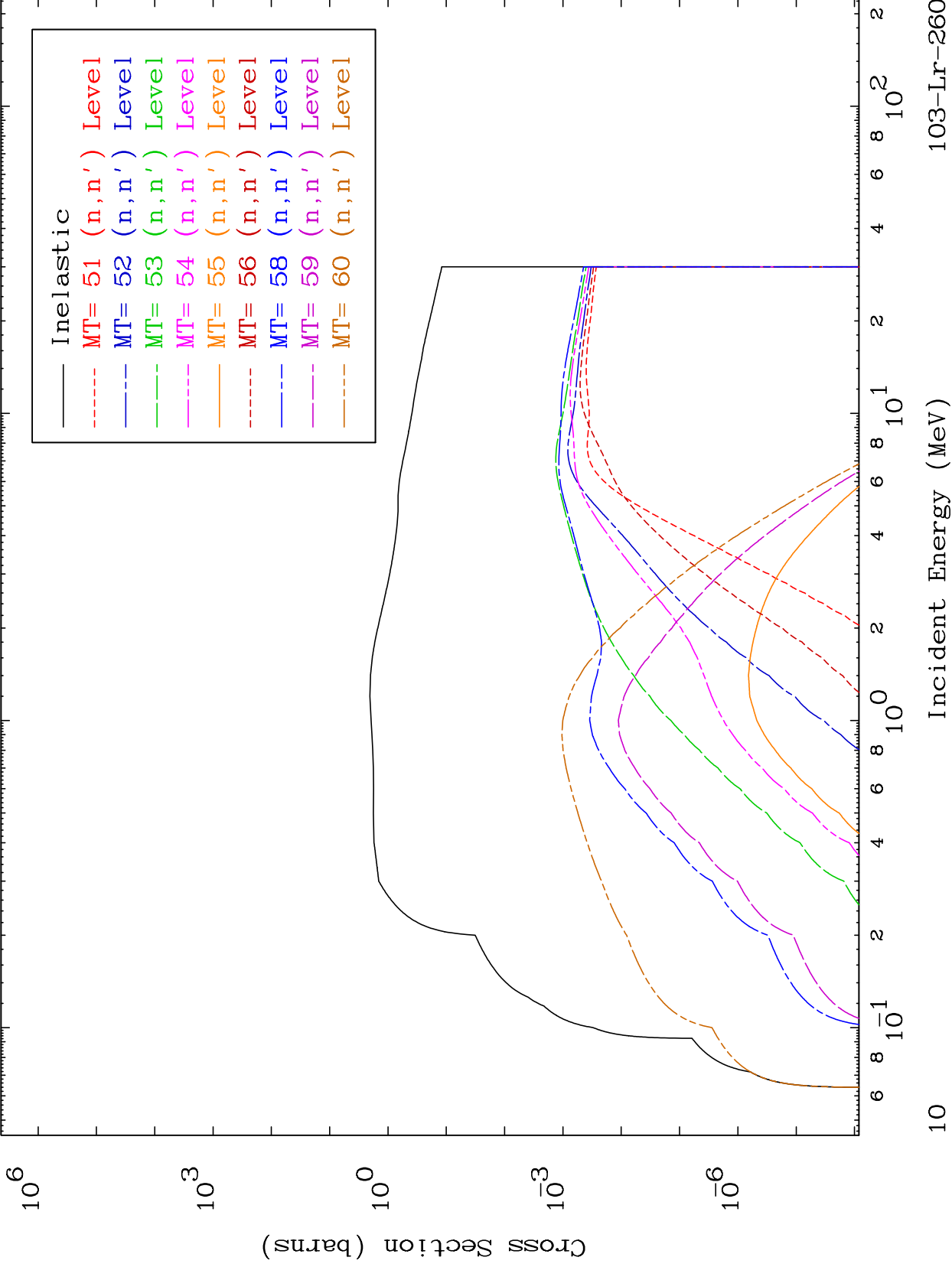


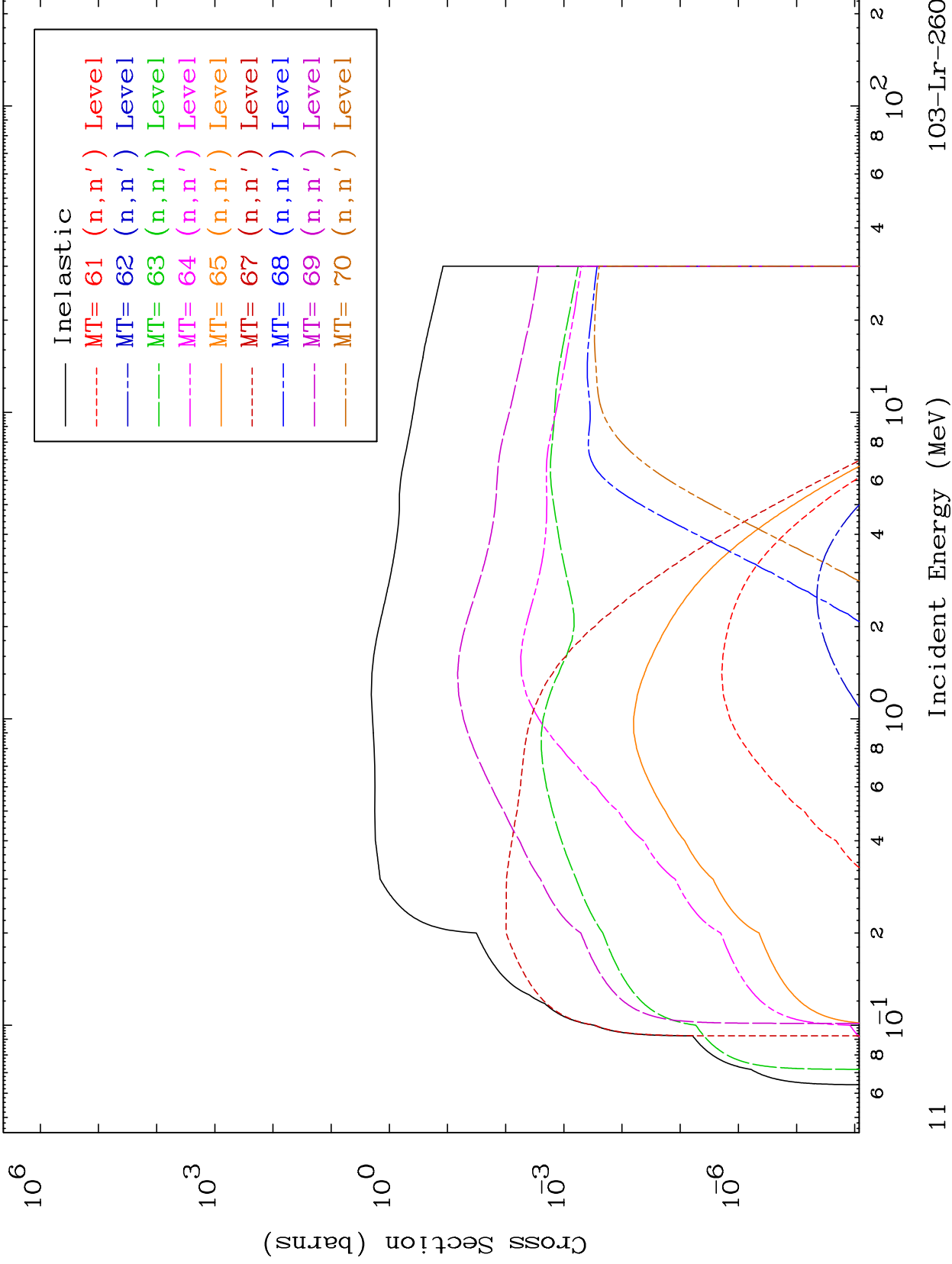
MAT 9989

(n,n') Levels

103-Lr-260

293 Kelvin Cross Sections



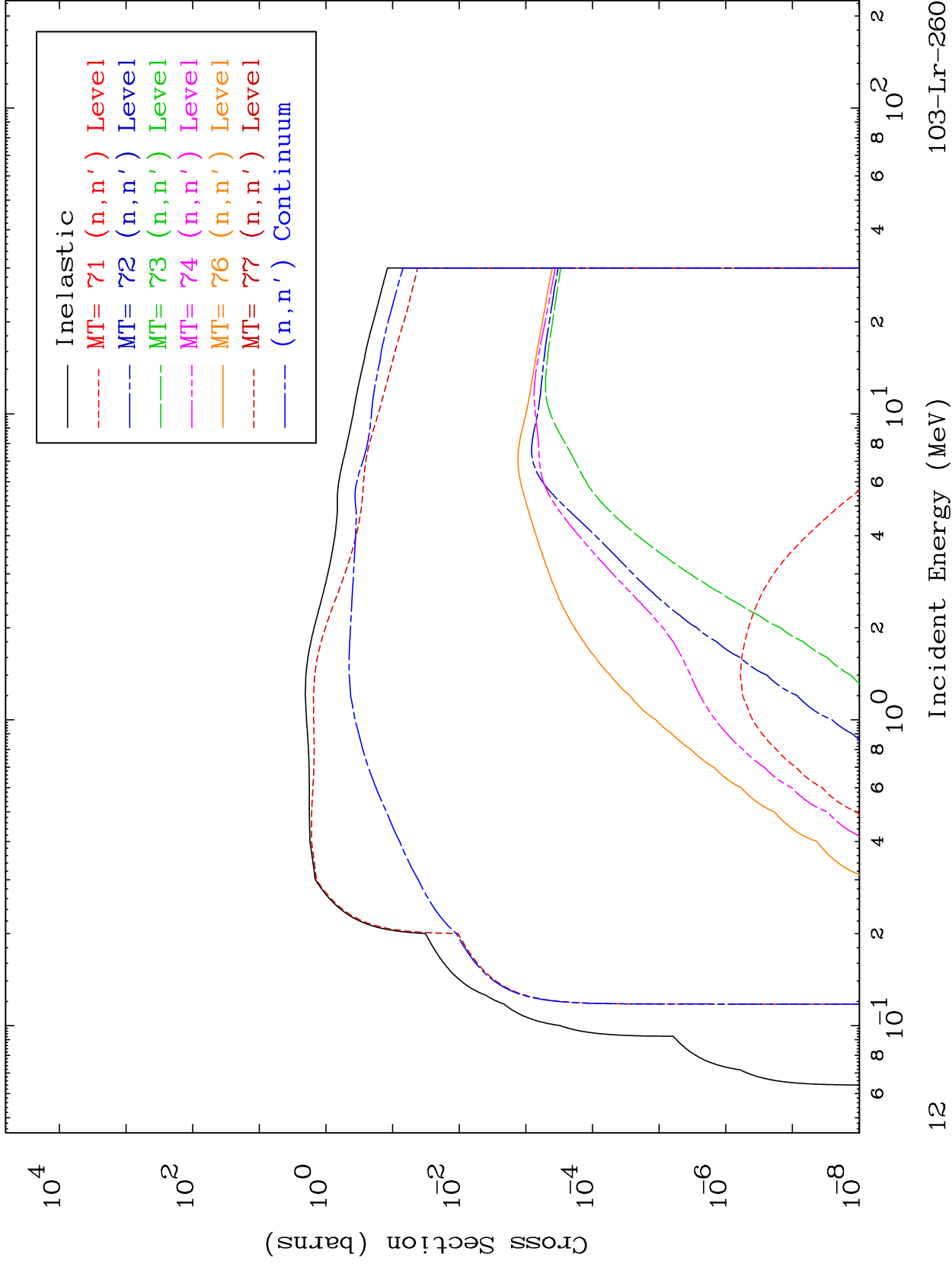


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(n,n') Levels

293 Kelvin Cross Sections

103-Lr-260

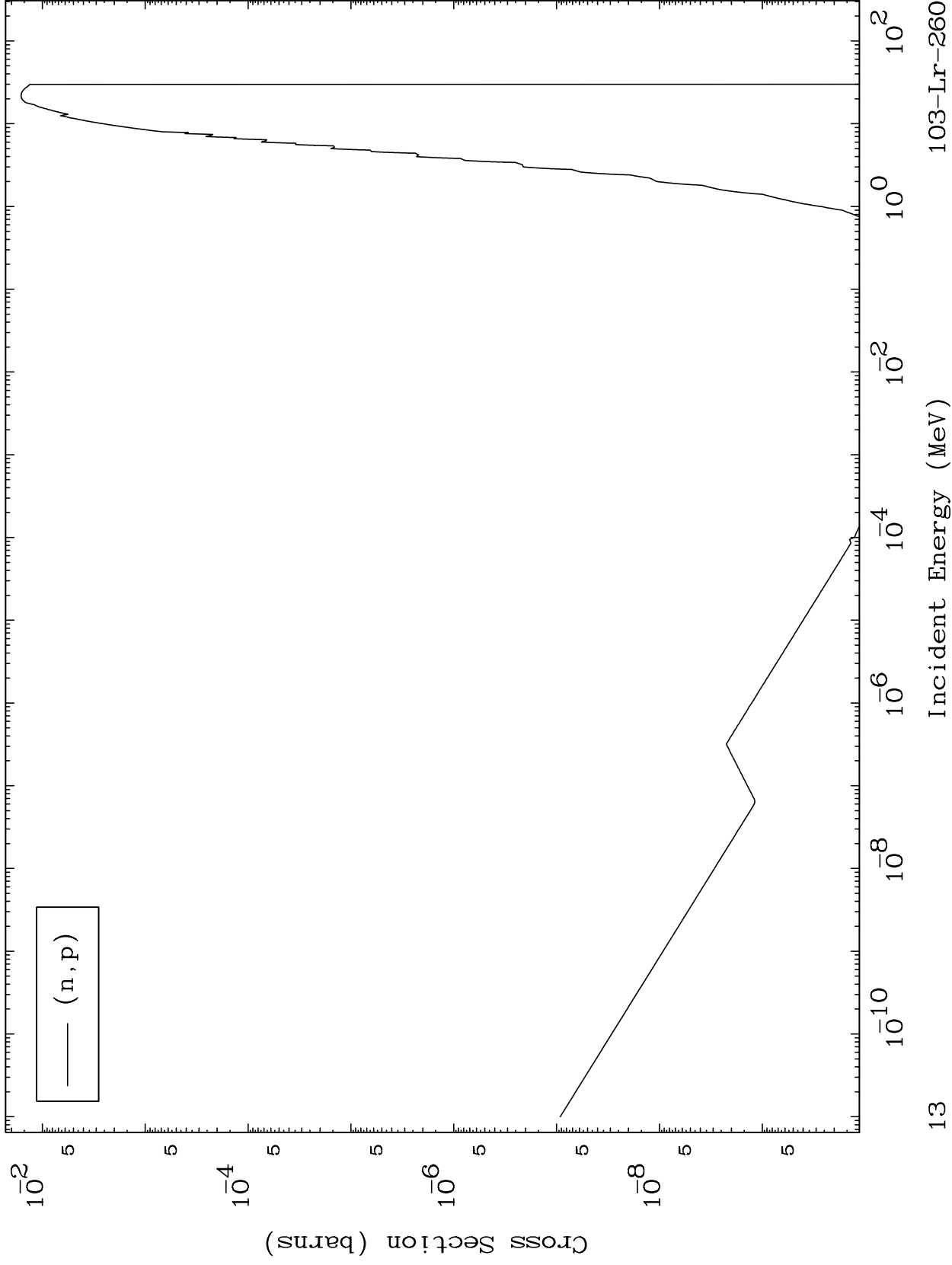


12

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(n,p) Levels  
293 Kelvin Cross Sections

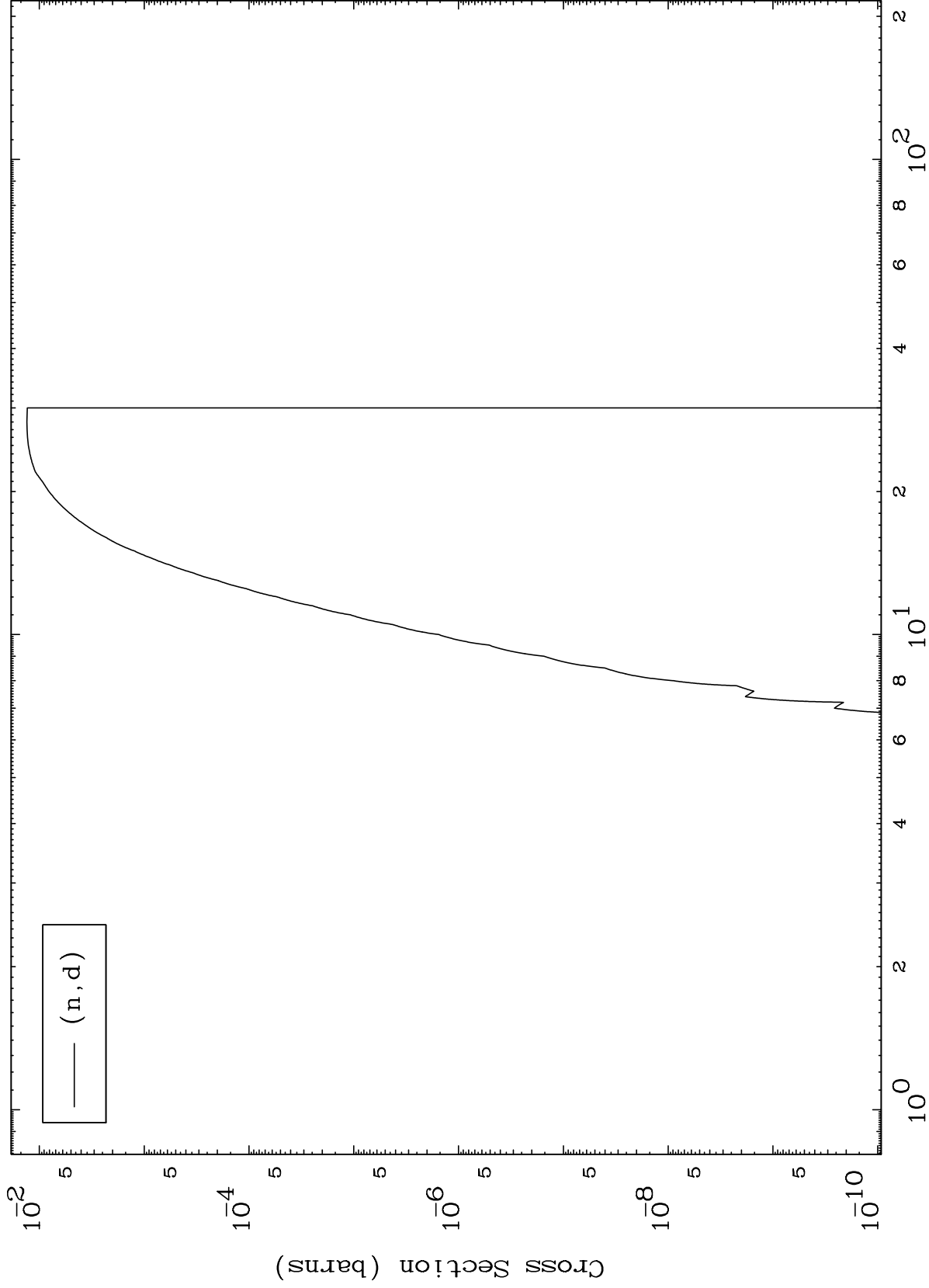
103-Lr-260



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(n,d) Levels  
293 Kelvin Cross Sections

103-Lr-260



14

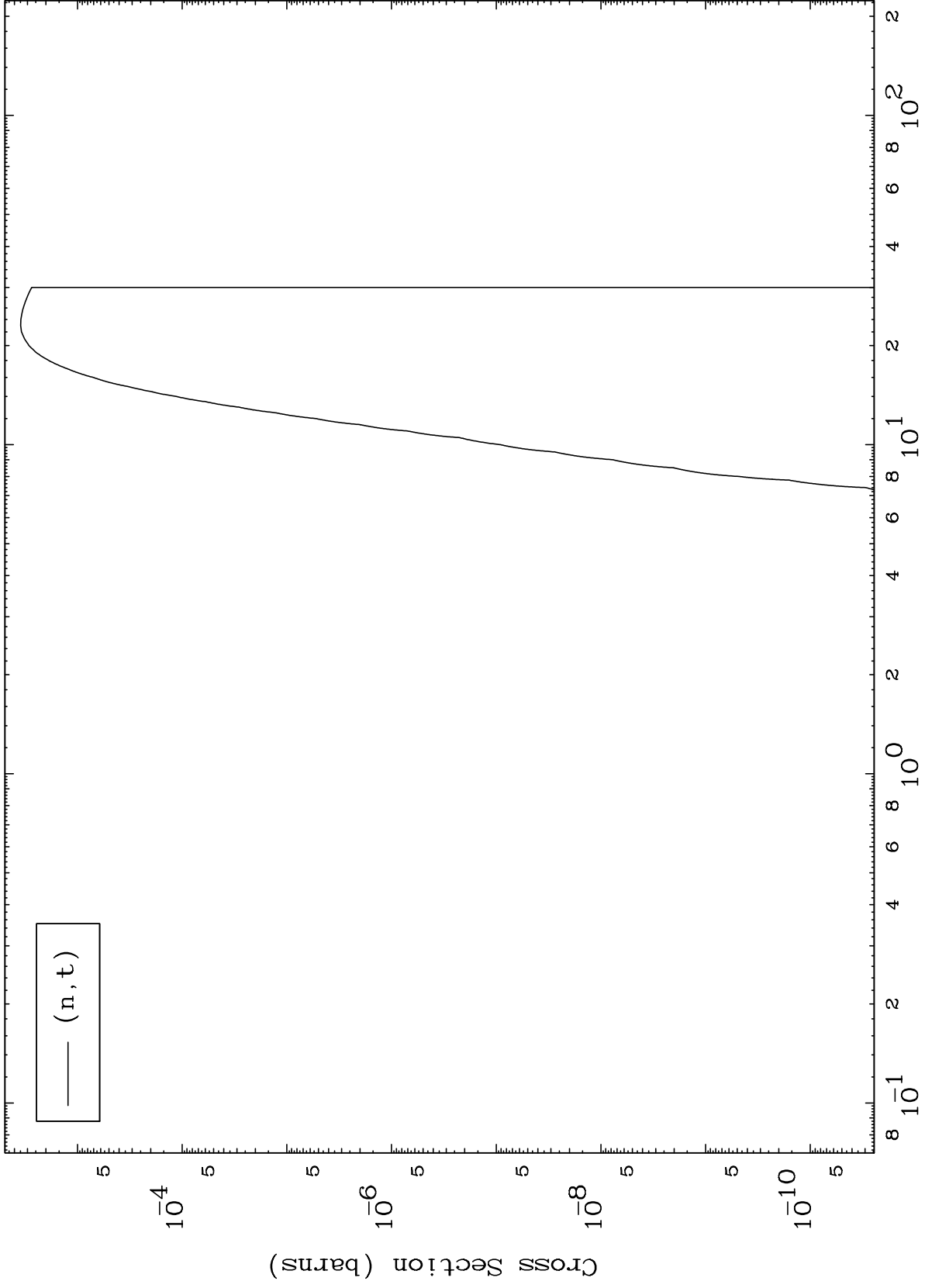
Incident Energy (MeV)

103-Lr-260

MAT 9989

(n,t) Levels  
293 Kelvin Cross Sections

103-Lr-260



15

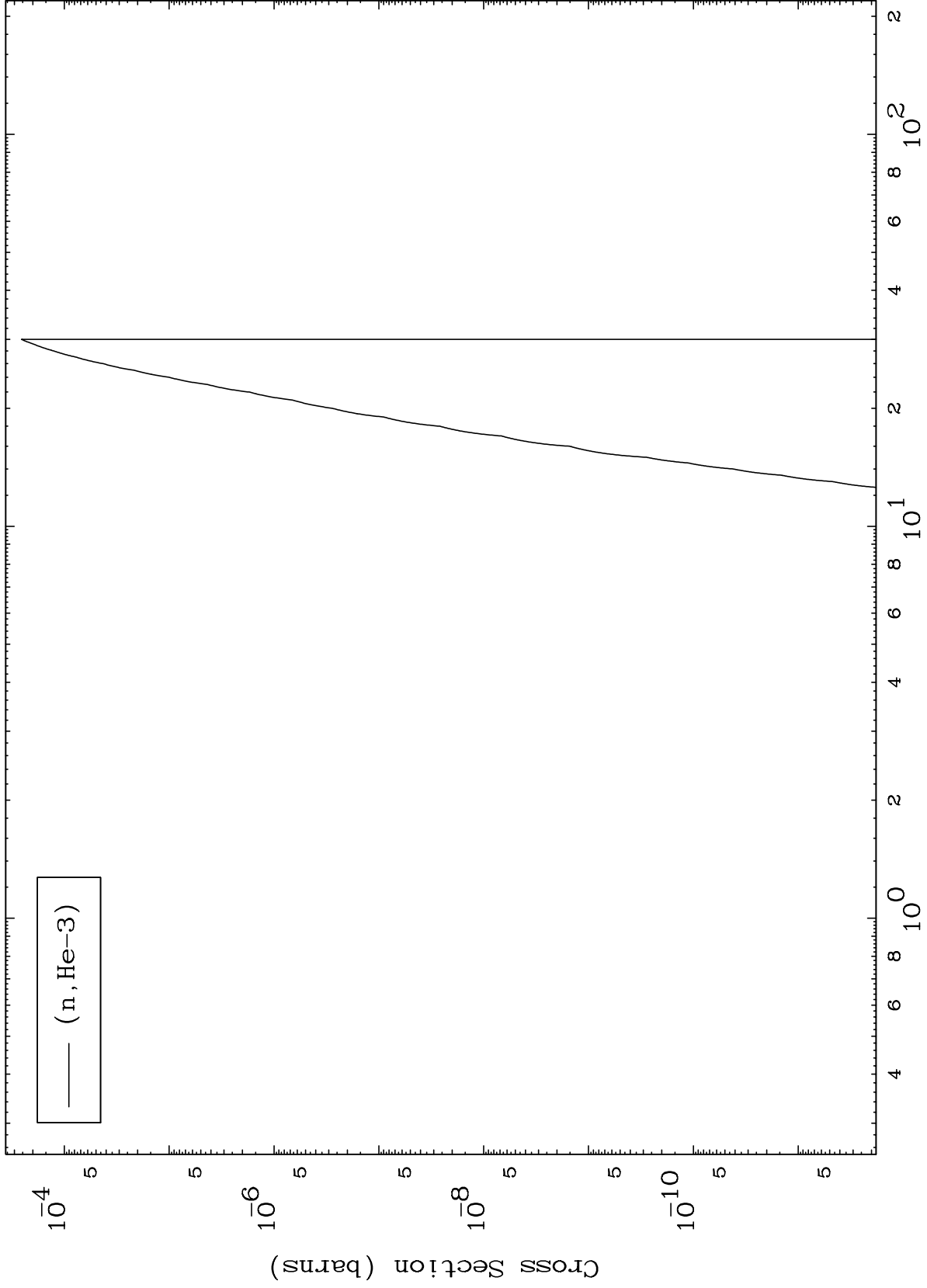
Incident Energy (MeV)

103-Lr-260

MAT 9989

(n,He3) Levels  
293 Kelvin Cross Sections

103-Lr-260



16

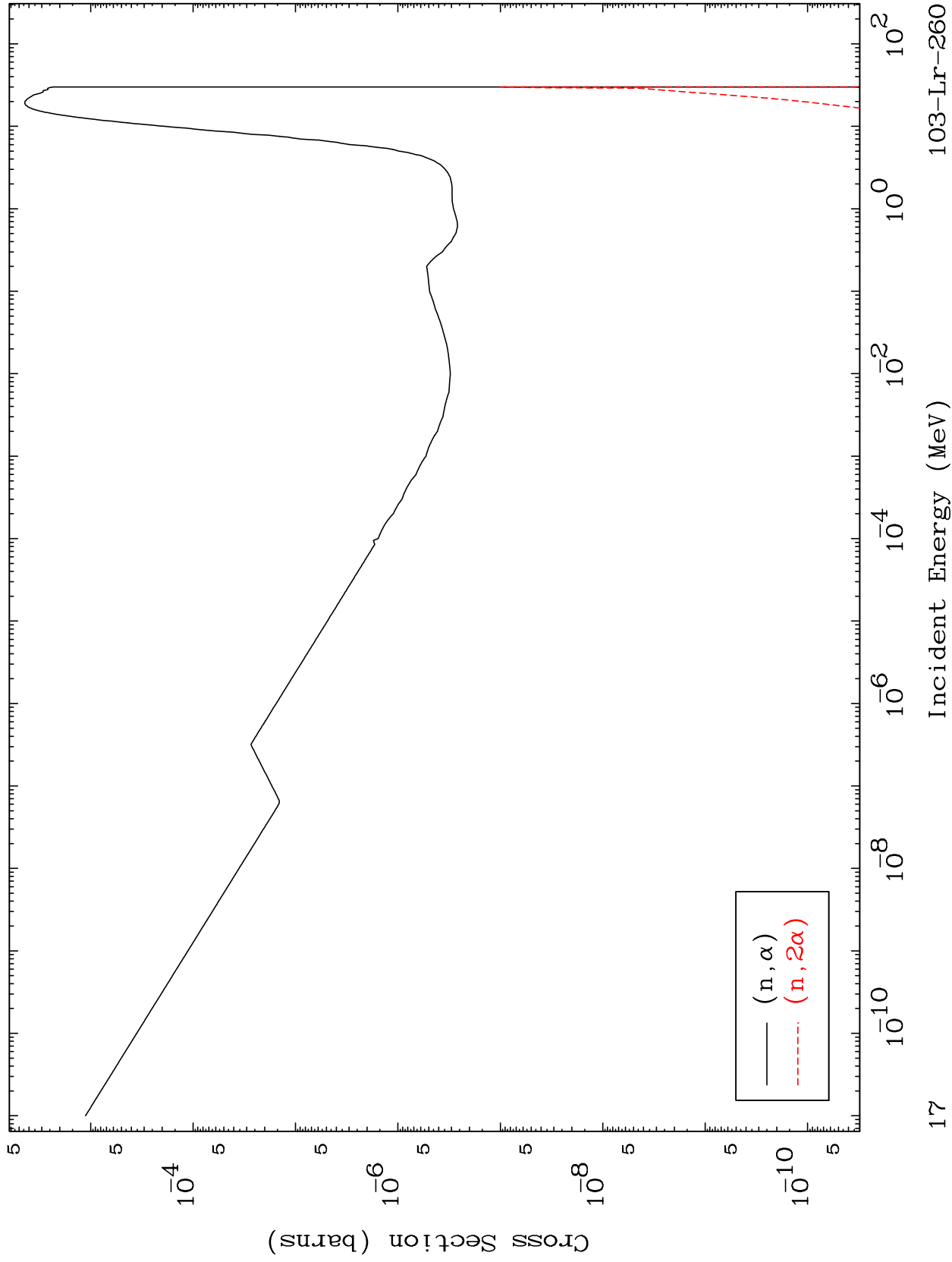
Incident Energy (MeV)

103-Lr-260

MAT 9989

(n,α) Levels  
293 Kelvin Cross Sections

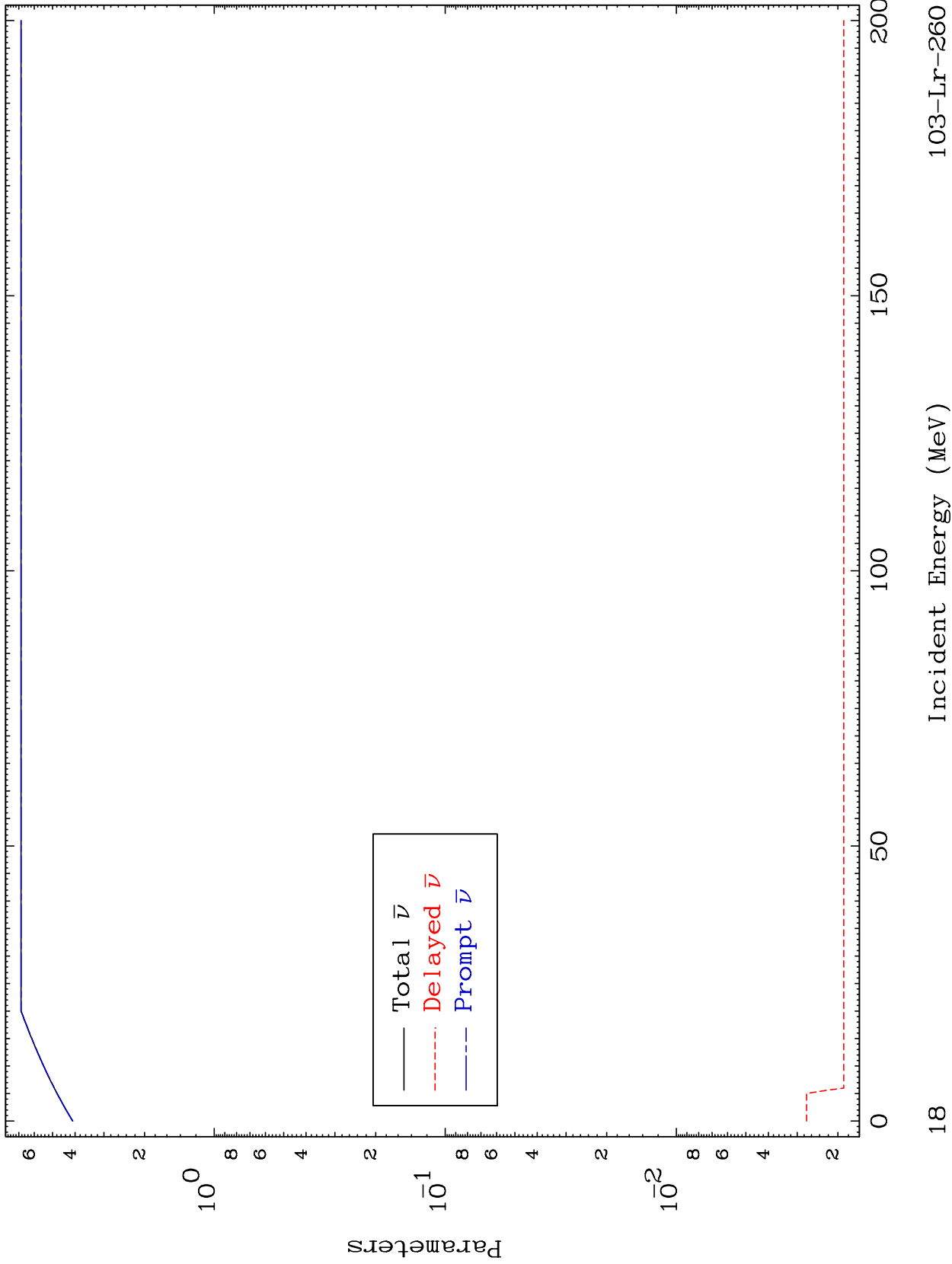
103-Lr-260



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Energy Release  
Parameters

103-Lr-260



18

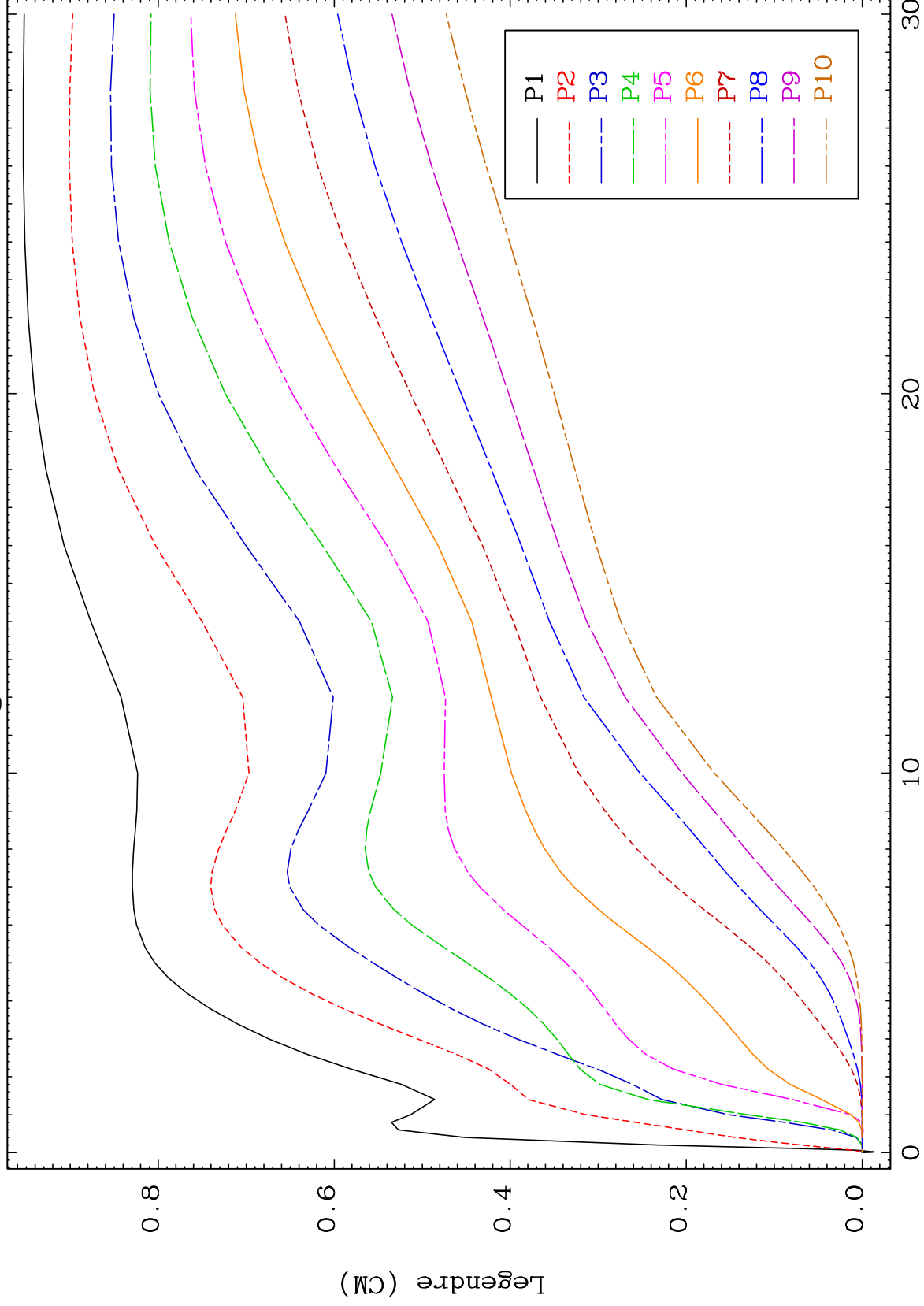
Incident Energy (MeV)

103-Lr-260

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Elastic Legendre Coefficients

103-Lr-260



19

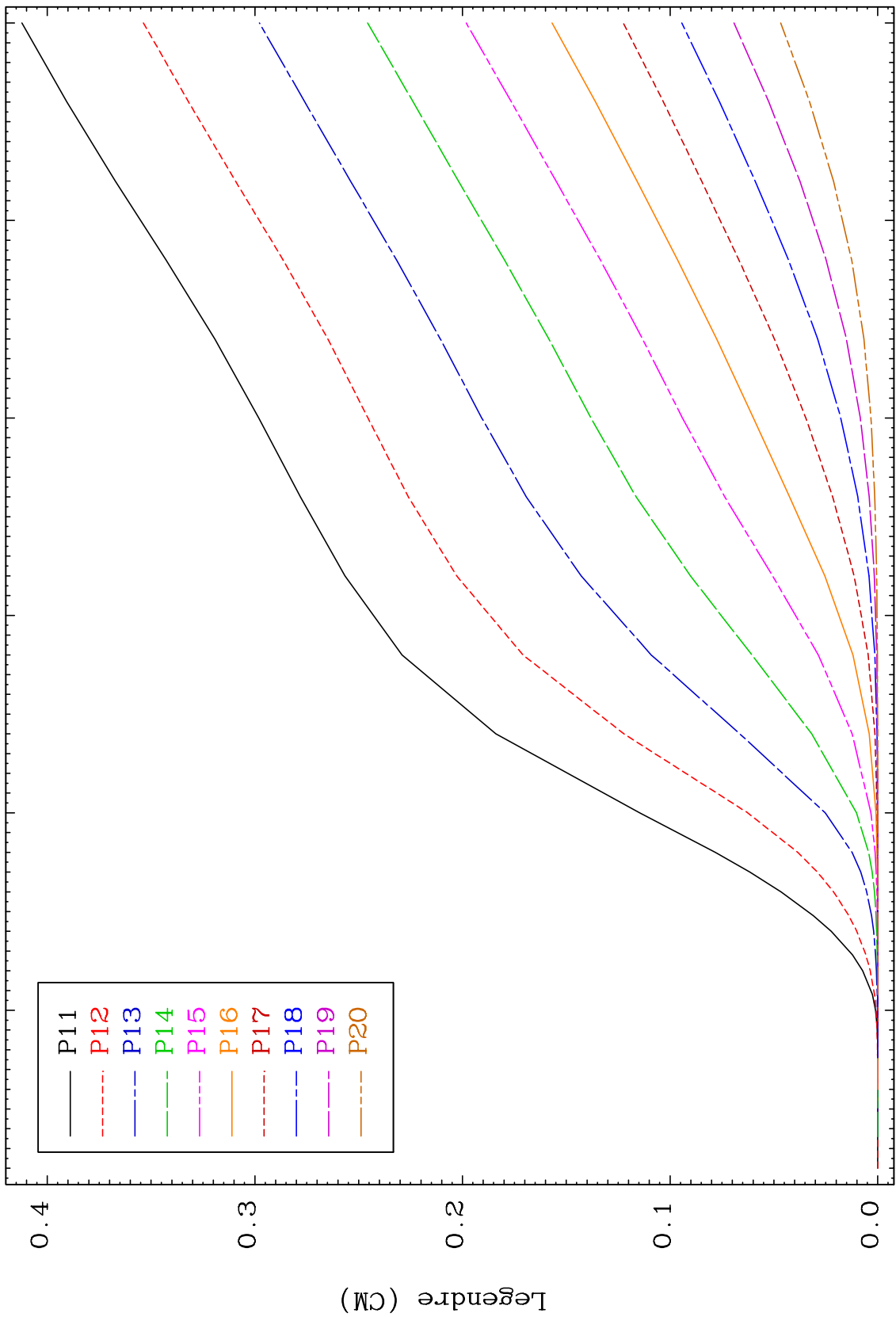
Incident Energy (MeV)

103-Lr-260

MAT 9989

Elastic Legendre Coefficients

103-Lr-260



20

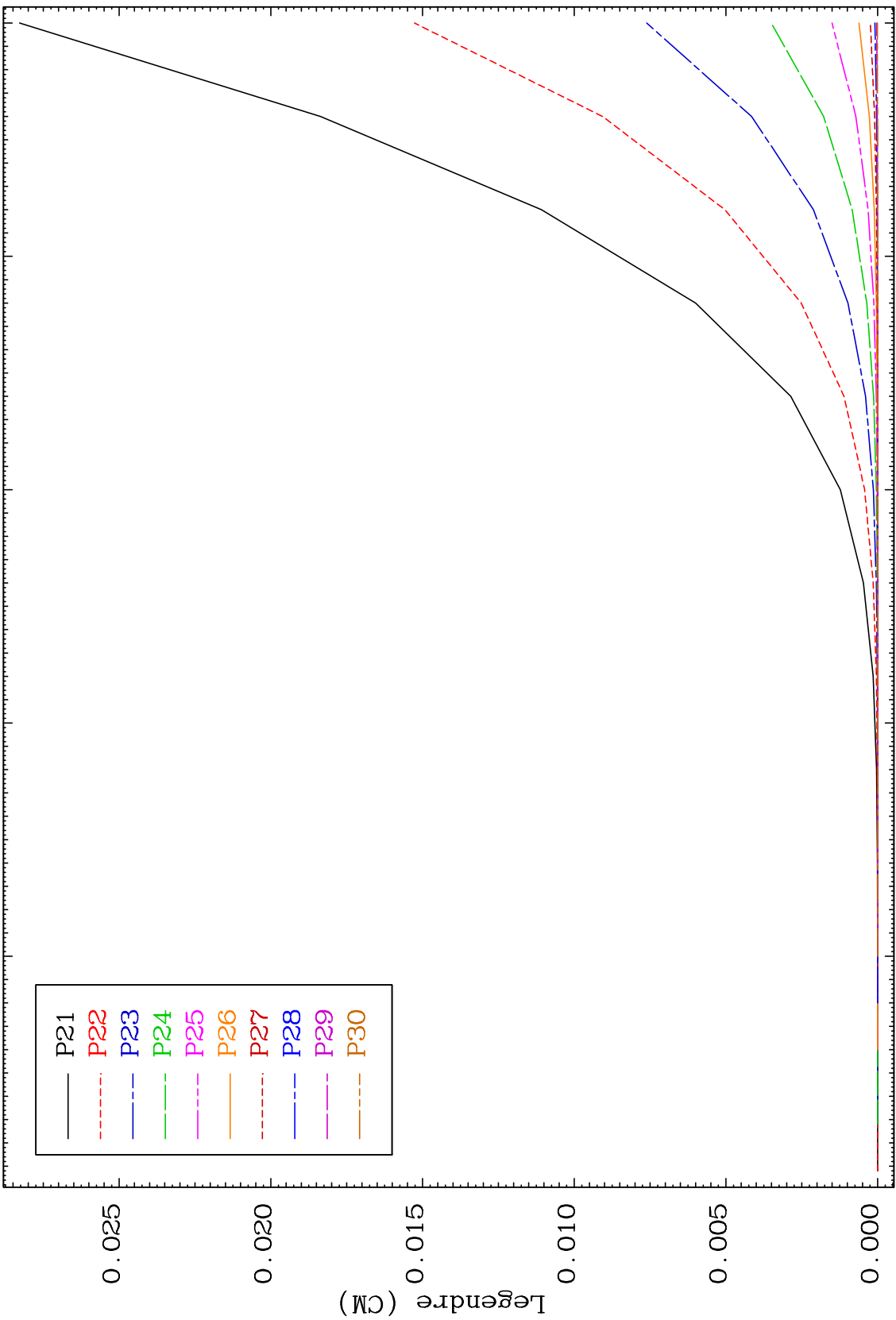
Incident Energy (MeV)

103-Lr-260

MAT 9989

Elastic Legendre Coefficients

103-Lr-260



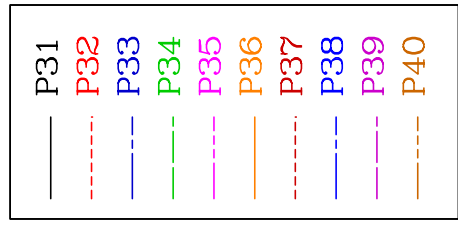
103-Lr-260

Incident Energy (MeV)

MAT 9989

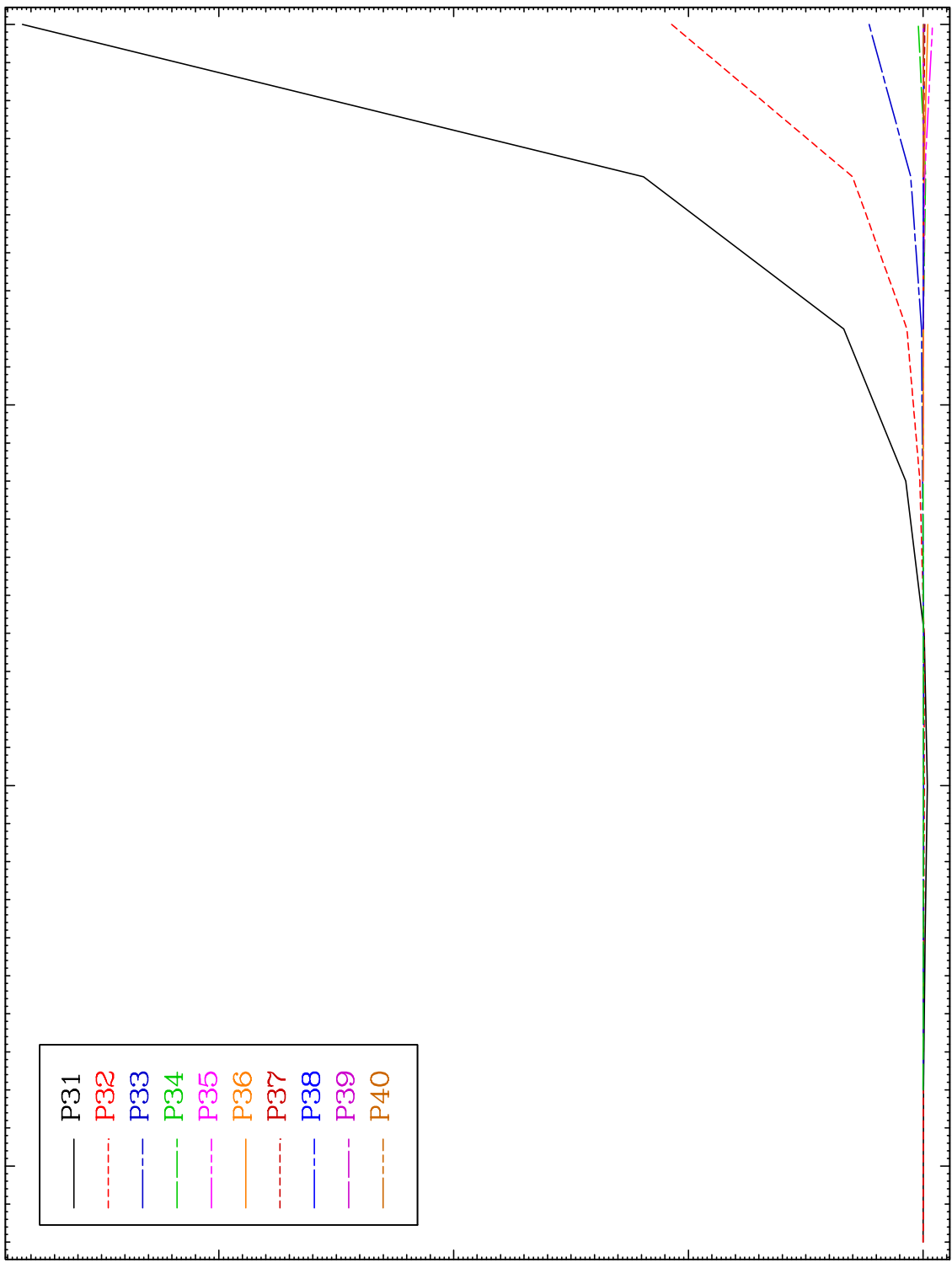
Elastic Legendre Coefficients

103-Lr-260



$\times 10^{-6}$

Legendre (CM)



30

25

20

15

22

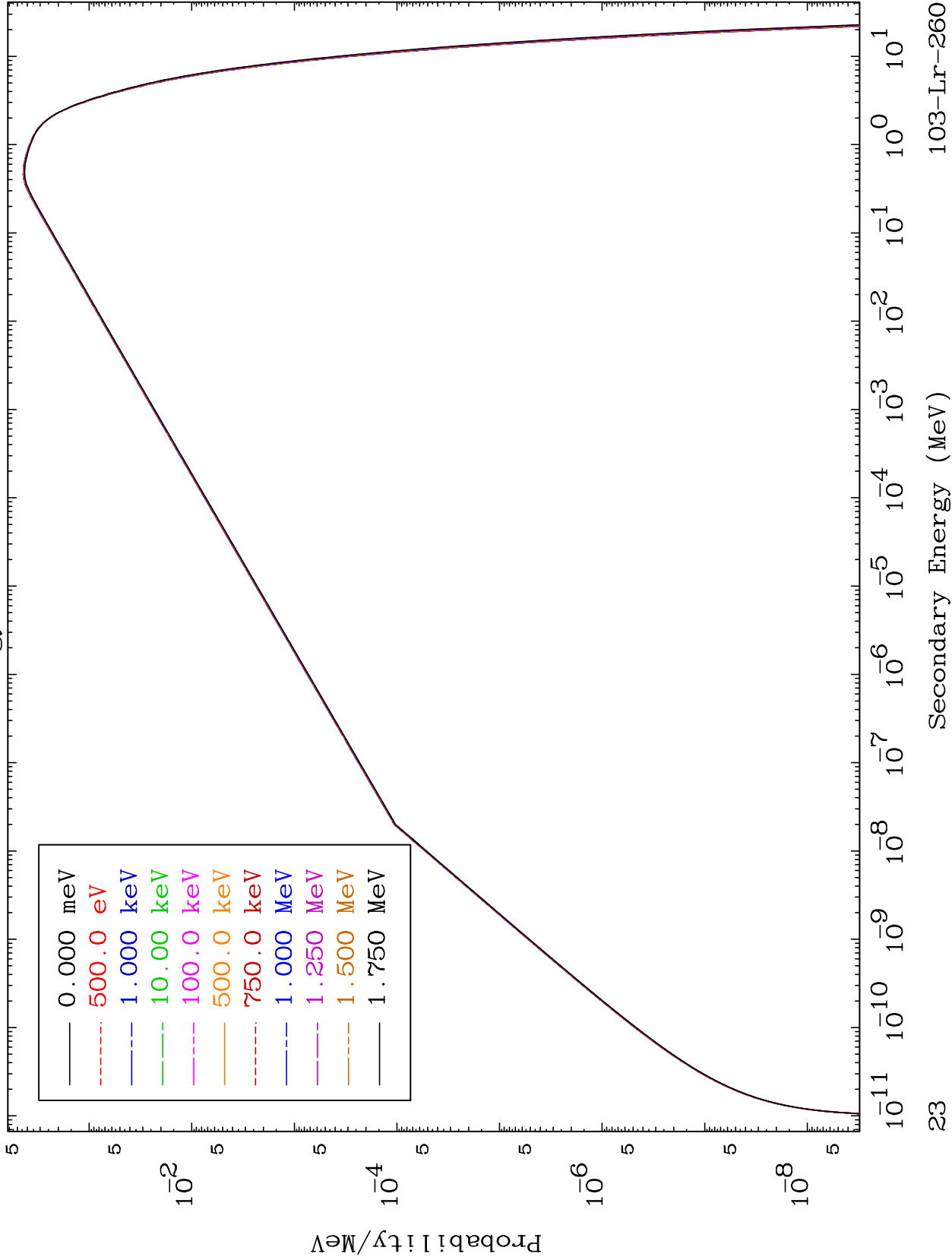
Incident Energy (MeV)

103-Lr-260

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### Fission Energy Distributions

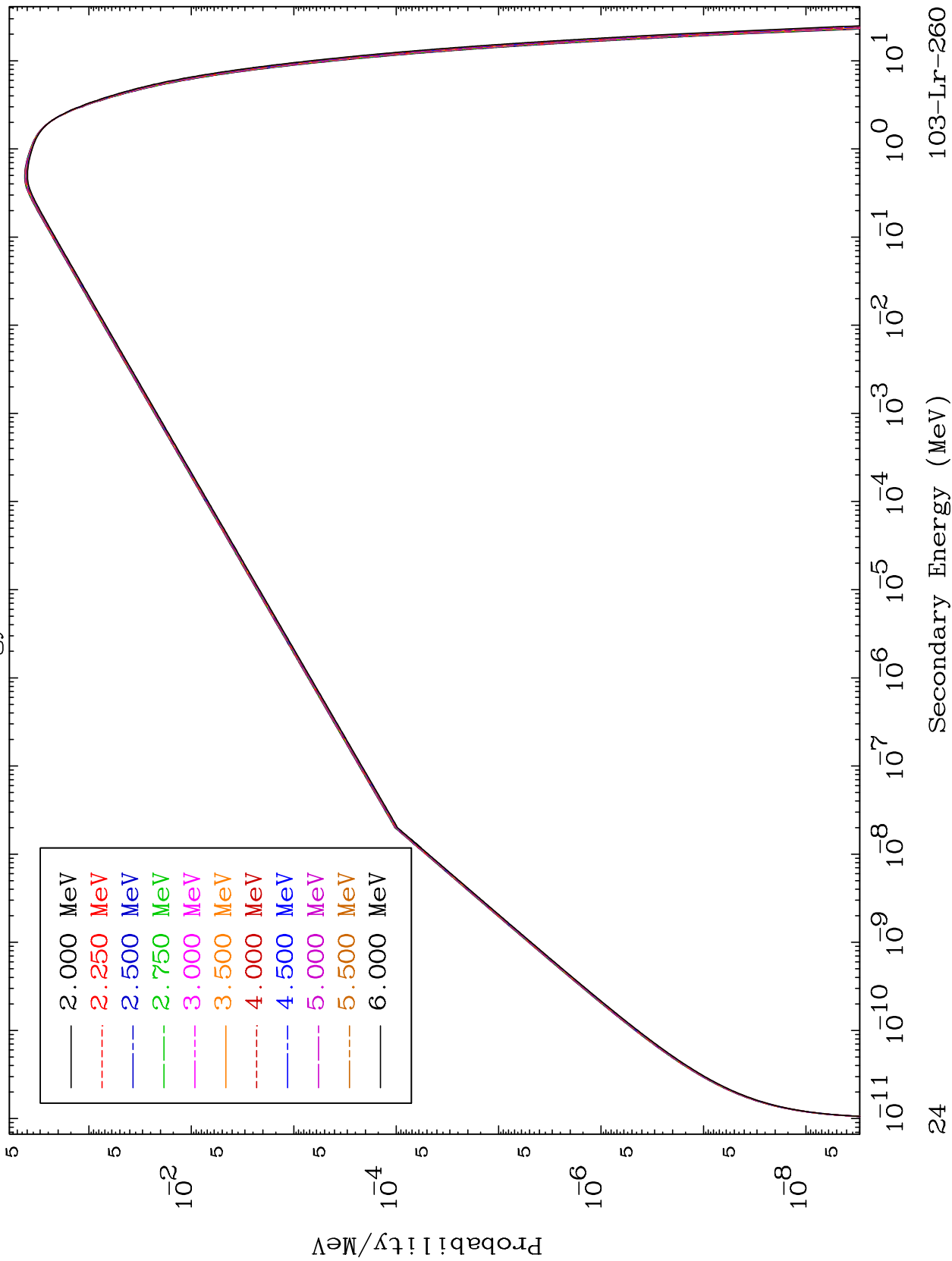
103-Lr-260



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### Fission Energy Distributions

103-Lr-260



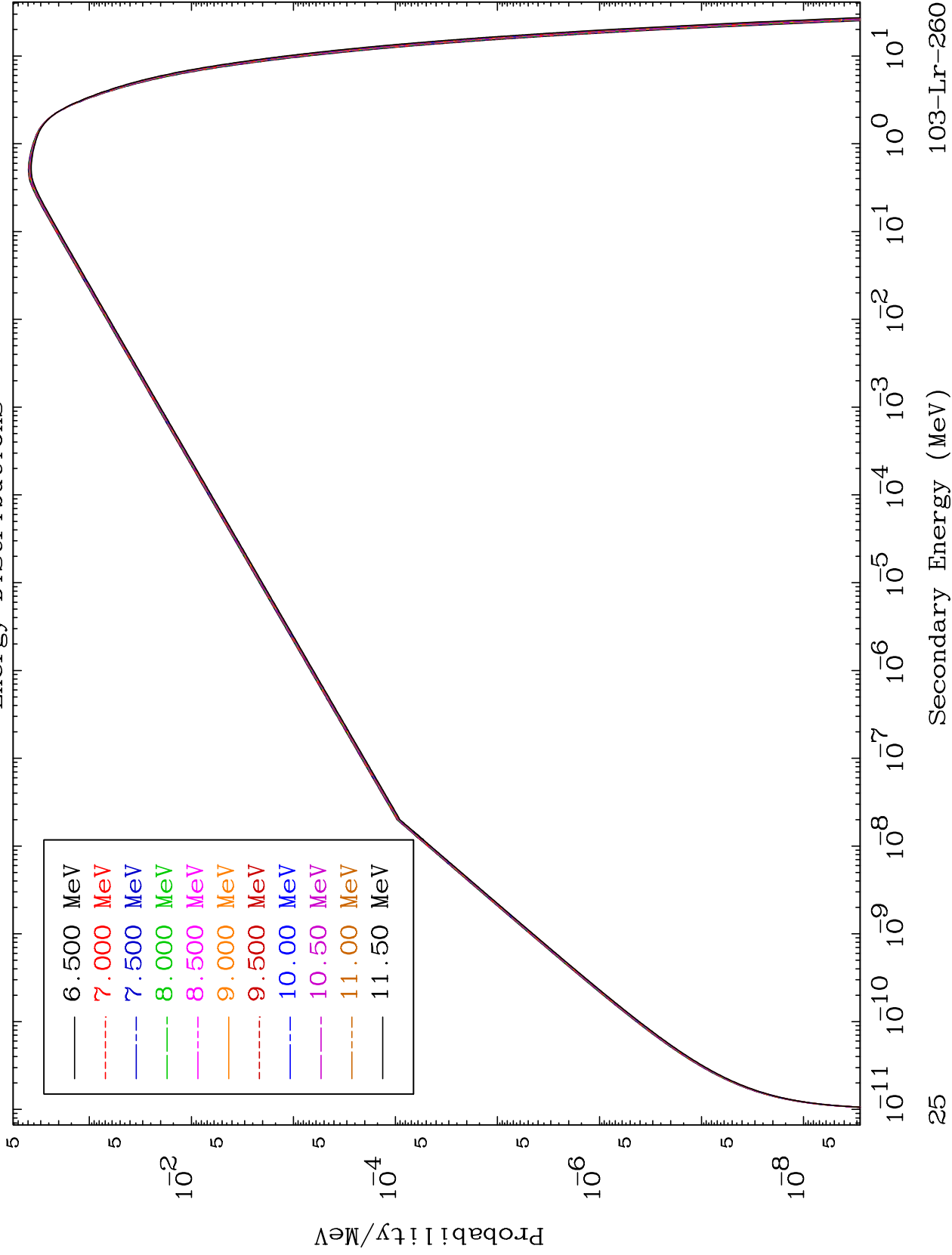
103-Lr-260

Secondary Energy (MeV)

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### Fission Energy Distributions

103-Lr-260

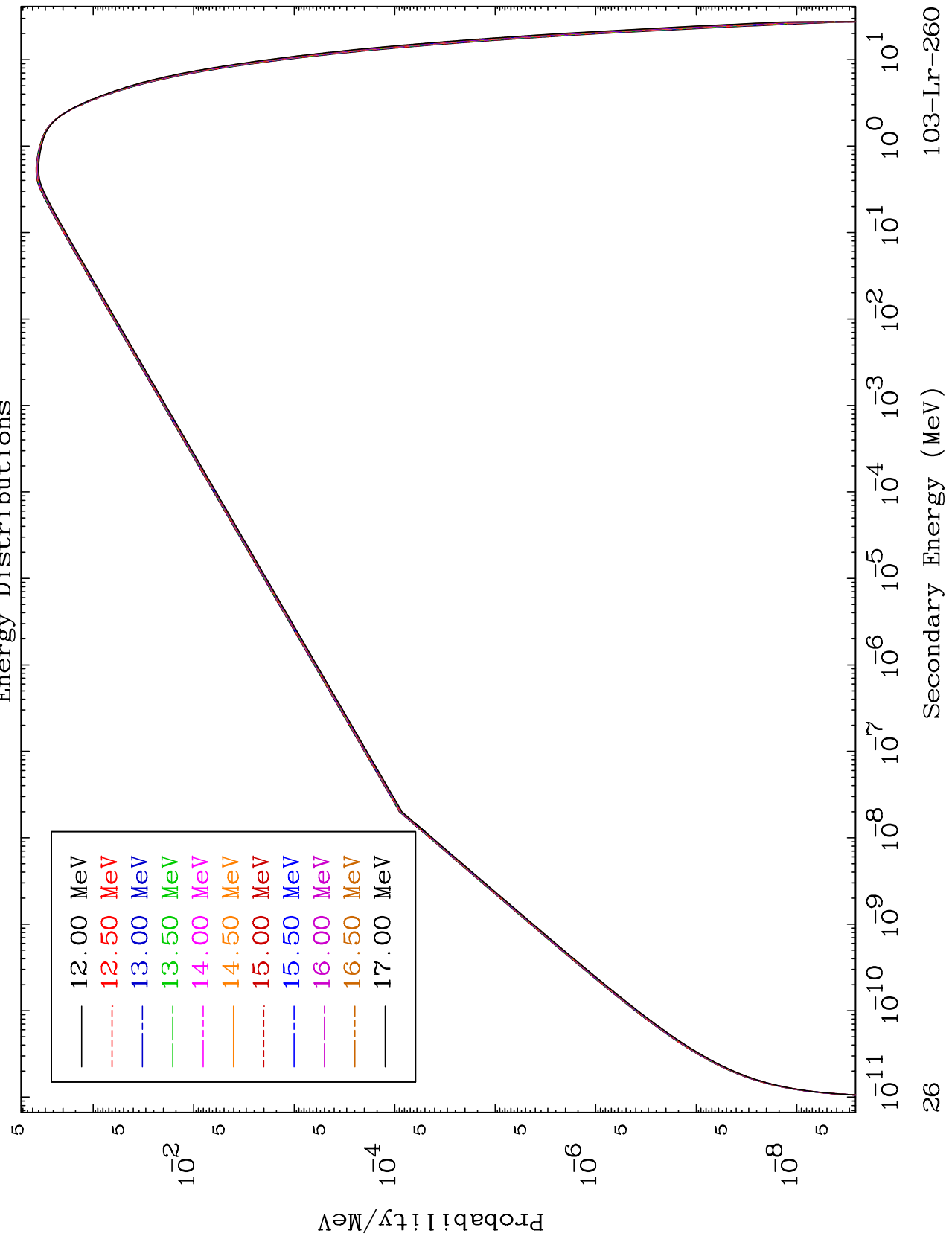


103-Lr-260

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### Fission Energy Distributions

103-Lr-260



103-Lr-260

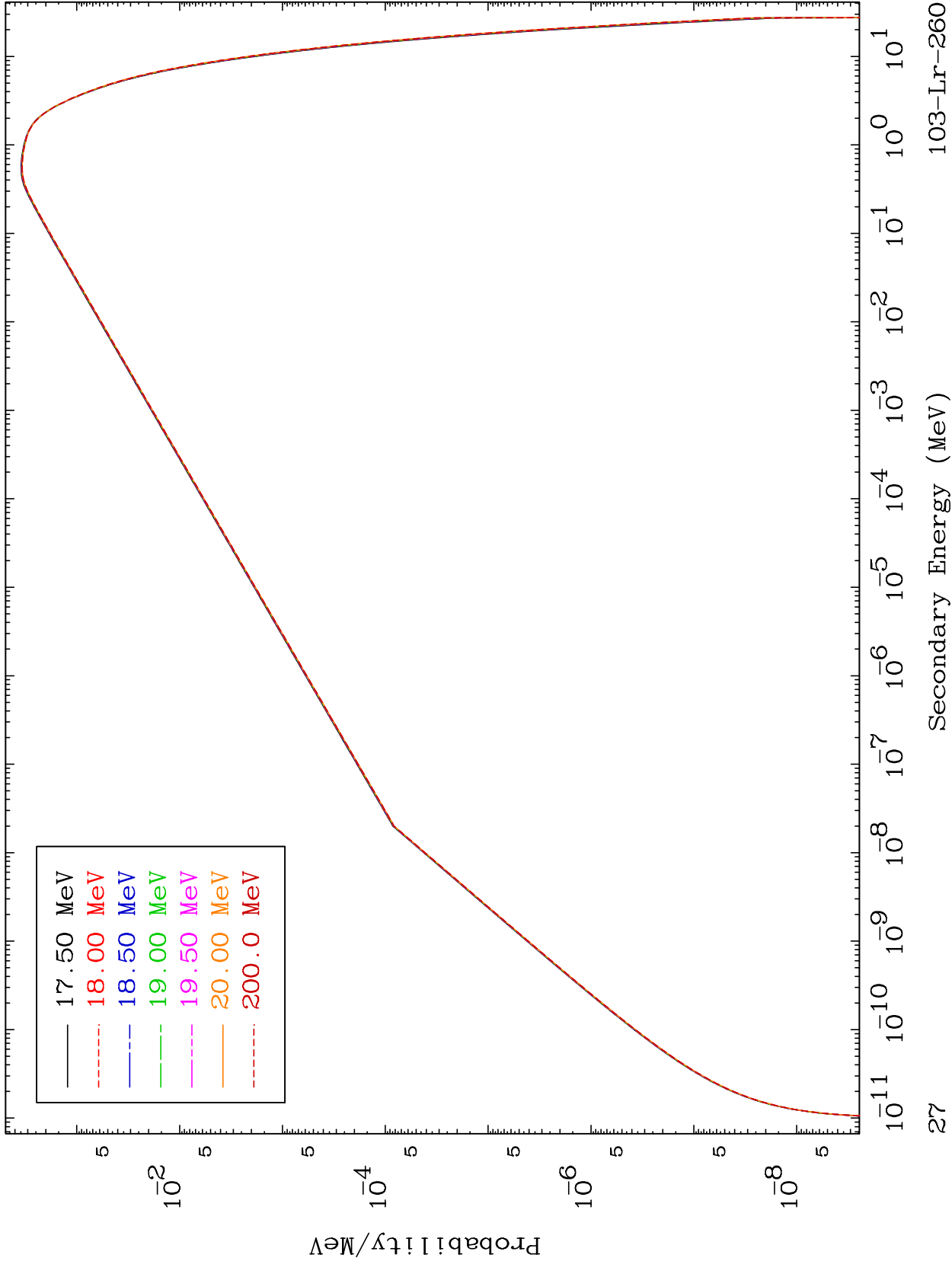
Secondary Energy (MeV)

26

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Fission Energy Distributions

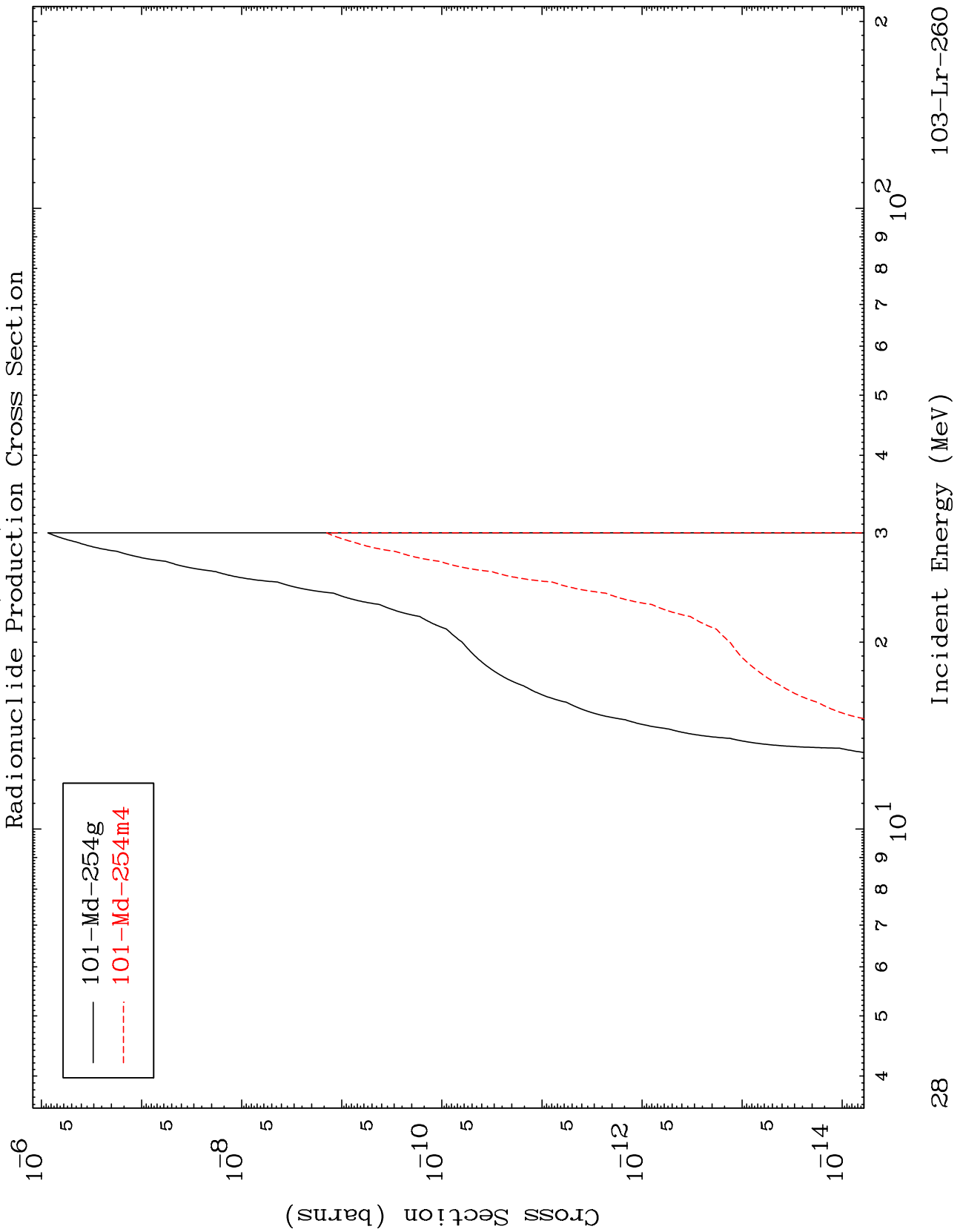
103-Lr-260



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(n,3n)  $\alpha$

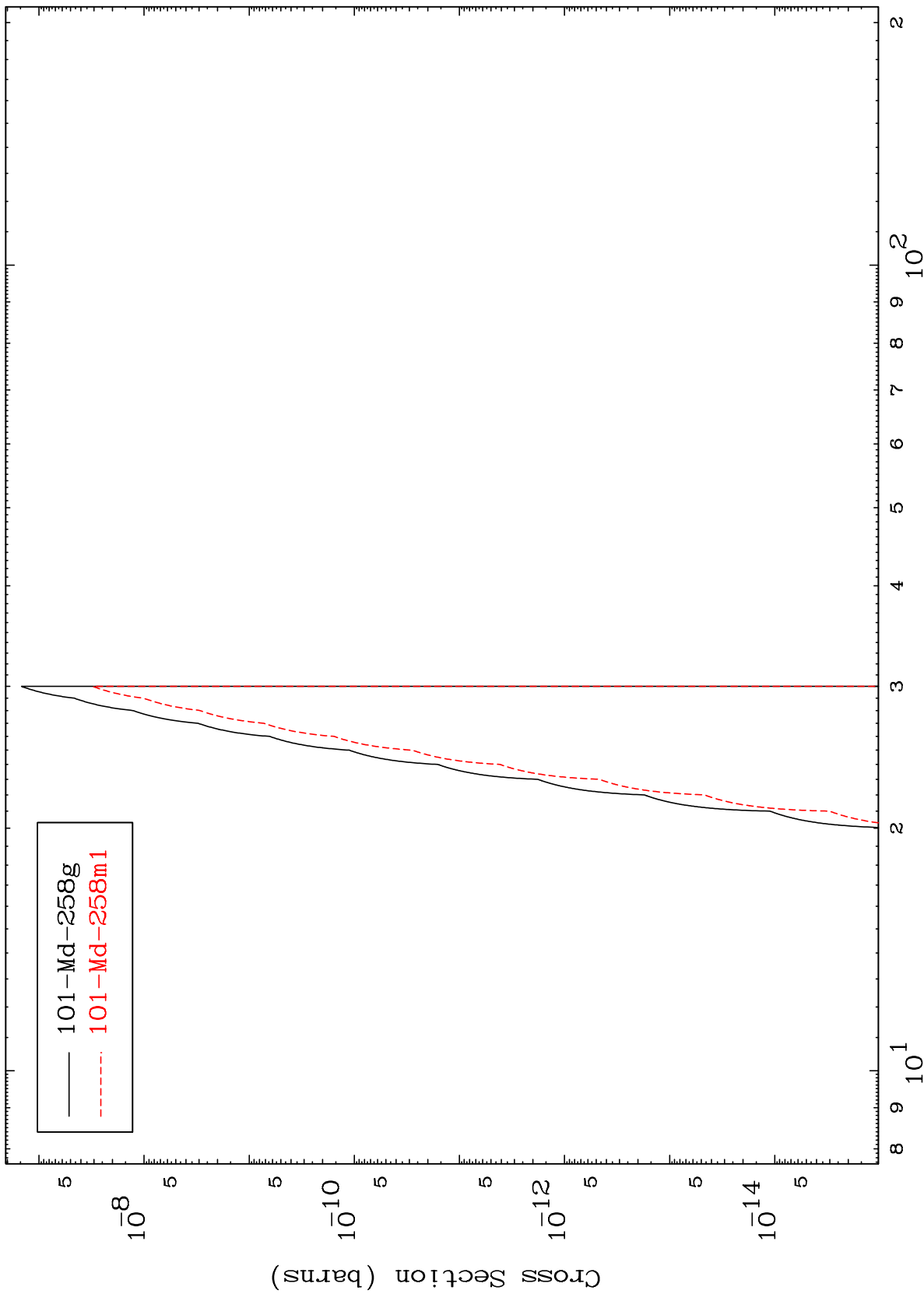
103-Lr-260



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103-Lr-260

(n,2n) p  
Radionuclide Production Cross Section



103-Lr-260

Incident Energy (MeV)

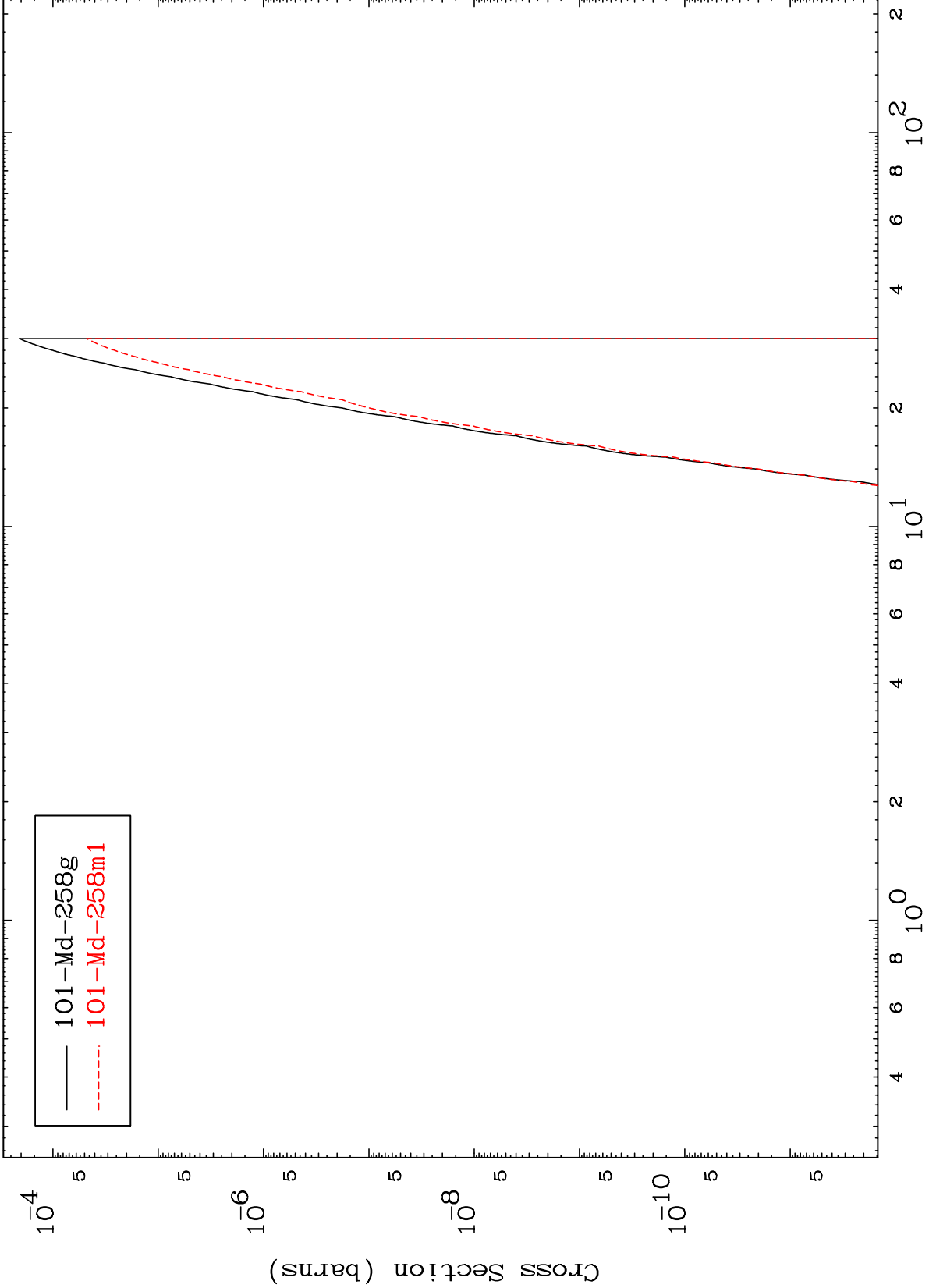
29

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(n,He-3)

103-Lr-260

Radionuclide Production Cross Section



MAT 9989

(n,p) d

103-Lr-260

Radionuclide Production Cross Section

