

Program EVALPLOT
(Version 2021-1)

by

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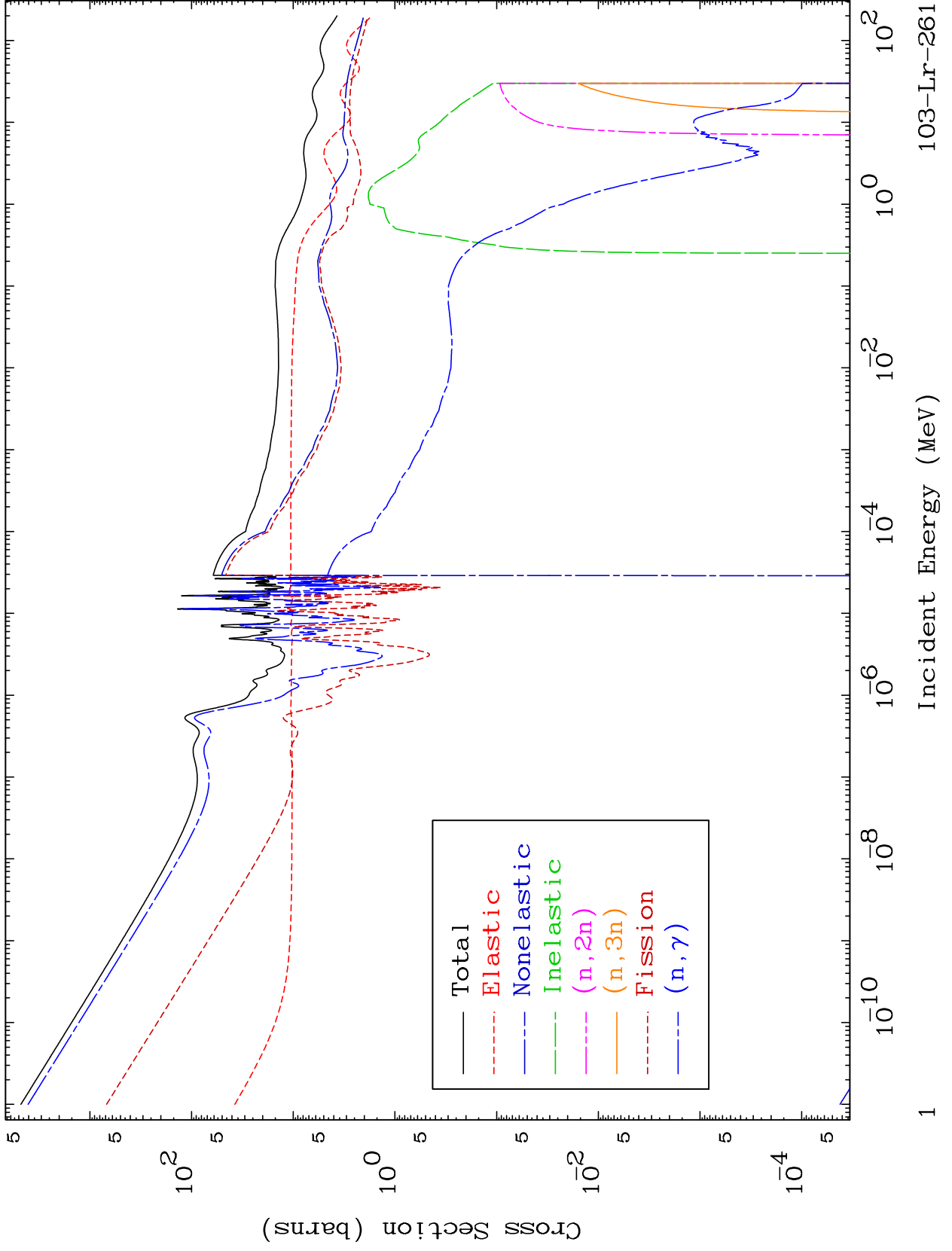
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Press Mouse Button to Start

MAT 9990

Neutron Major
293 Kelvin Cross Sections

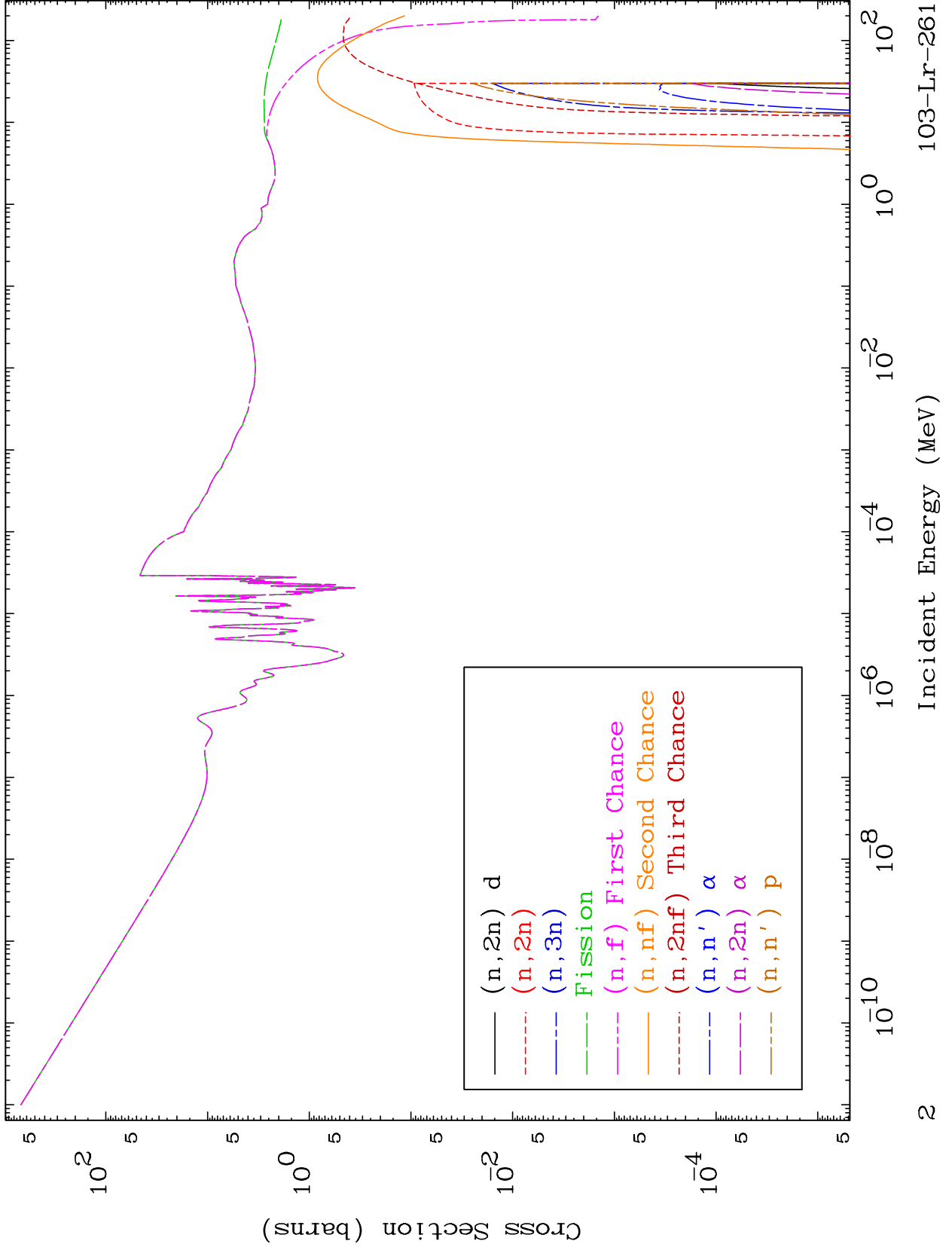
103-Lr-261

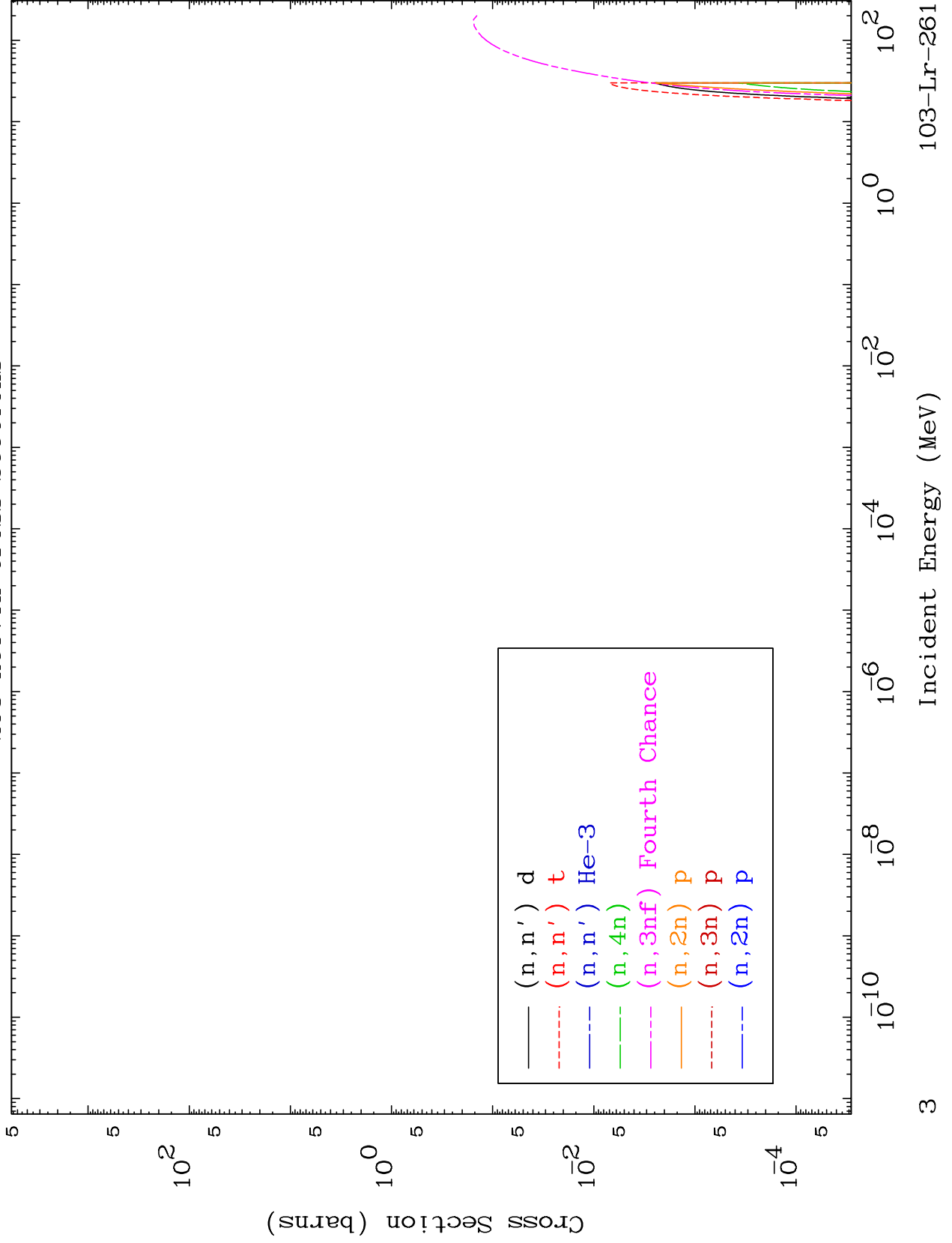


MAT 9990

Neutron Absorption
293 Kelvin Cross Sections

103-Lr-261

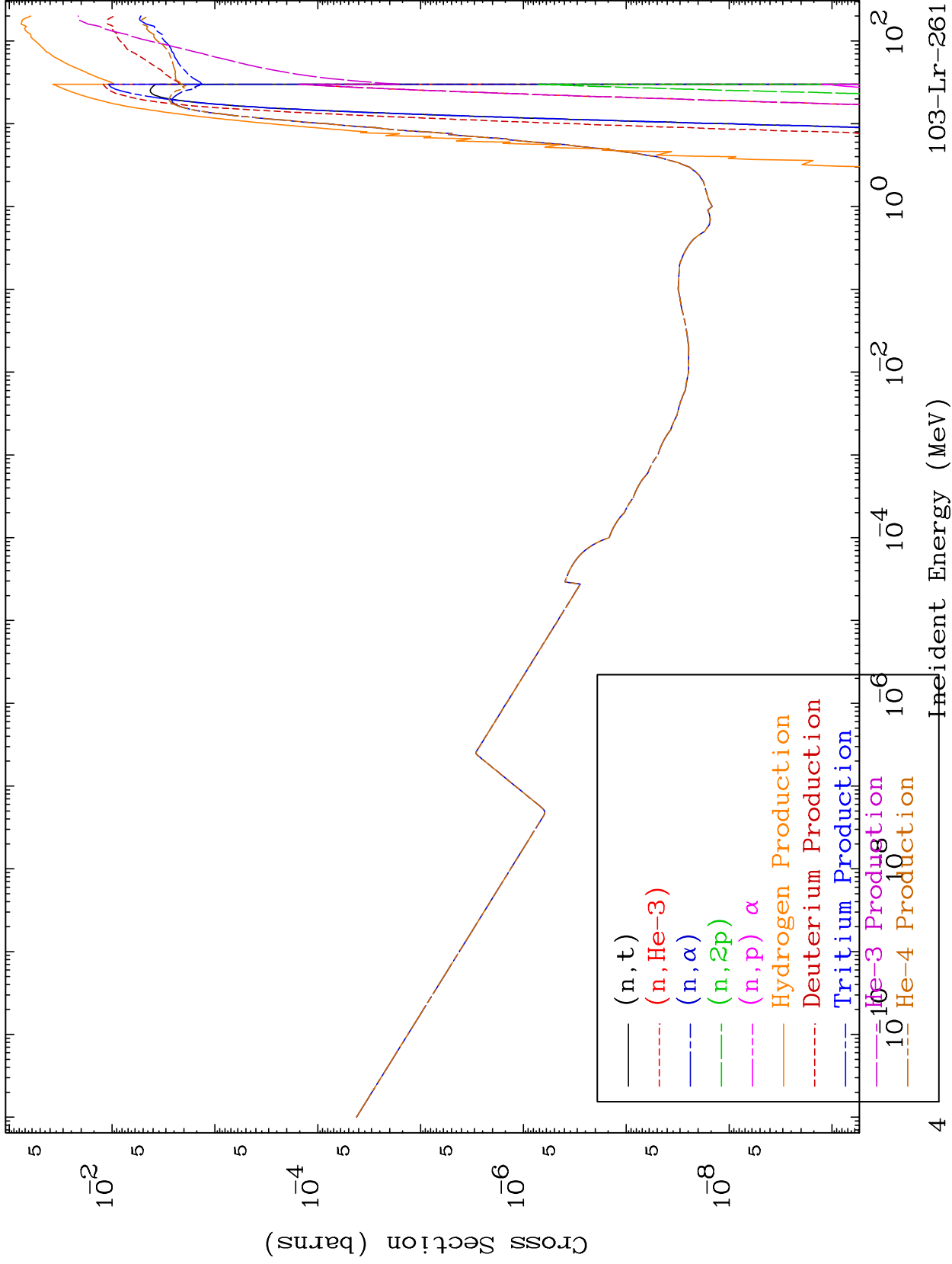




MAT 9990

Neutron Absorption
293 Kelvin Cross Sections

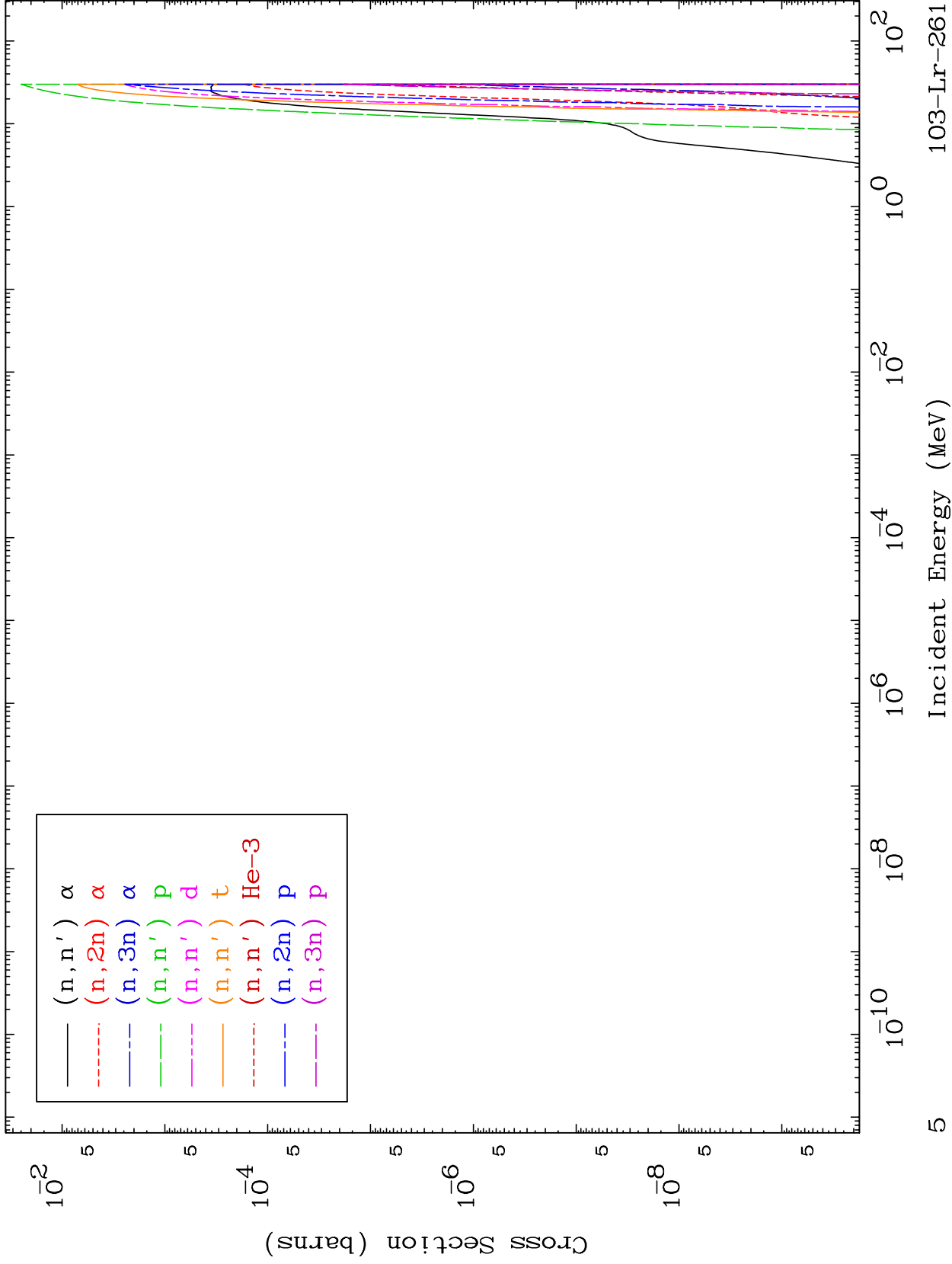
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MAT 9990

Charged Particle
293 Kelvin Cross Sections

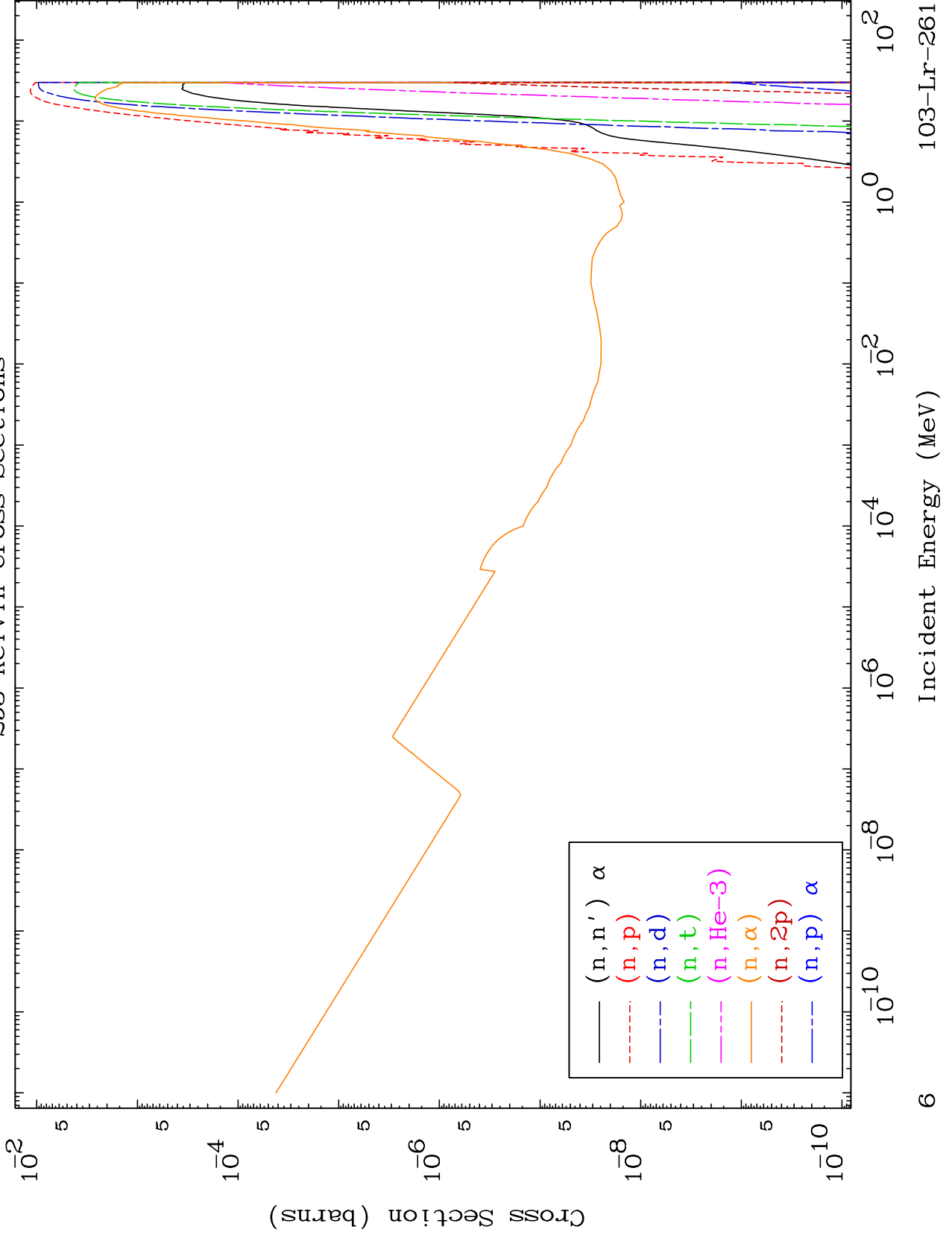
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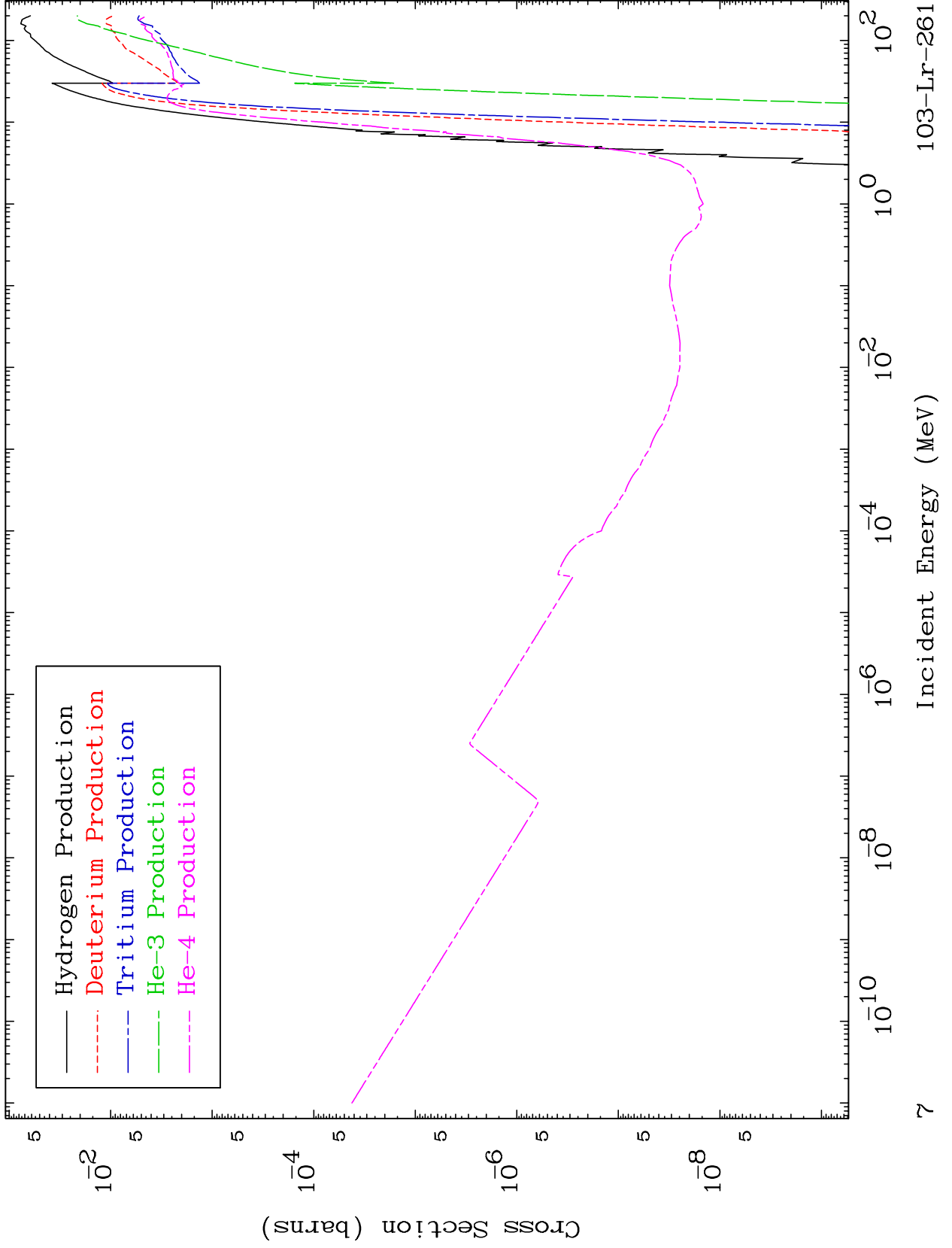


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Charged Particle
293 Kelvin Cross Sections

103-Lr-261

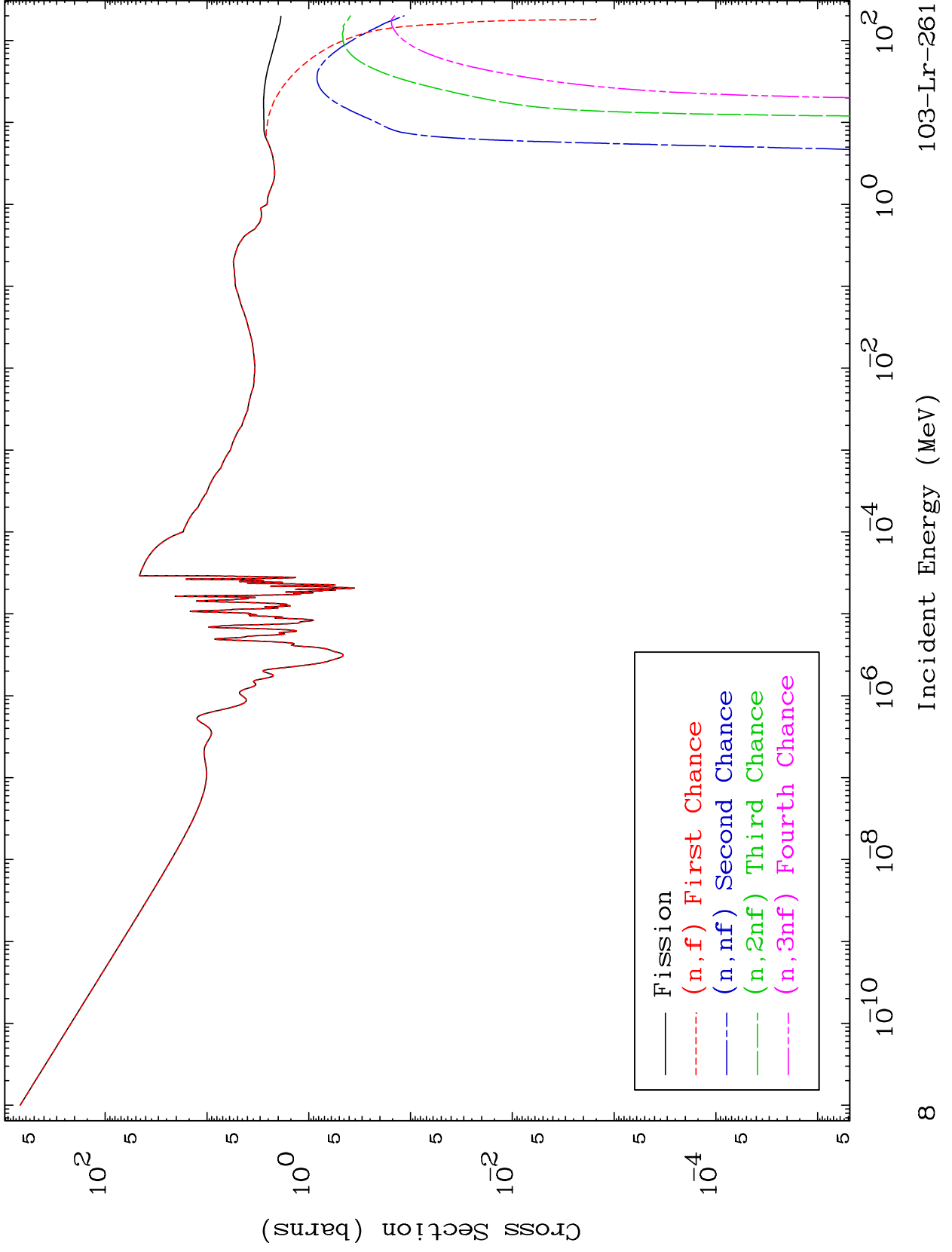




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Fission
293 Kelvin Cross Sections

103-Lr-261

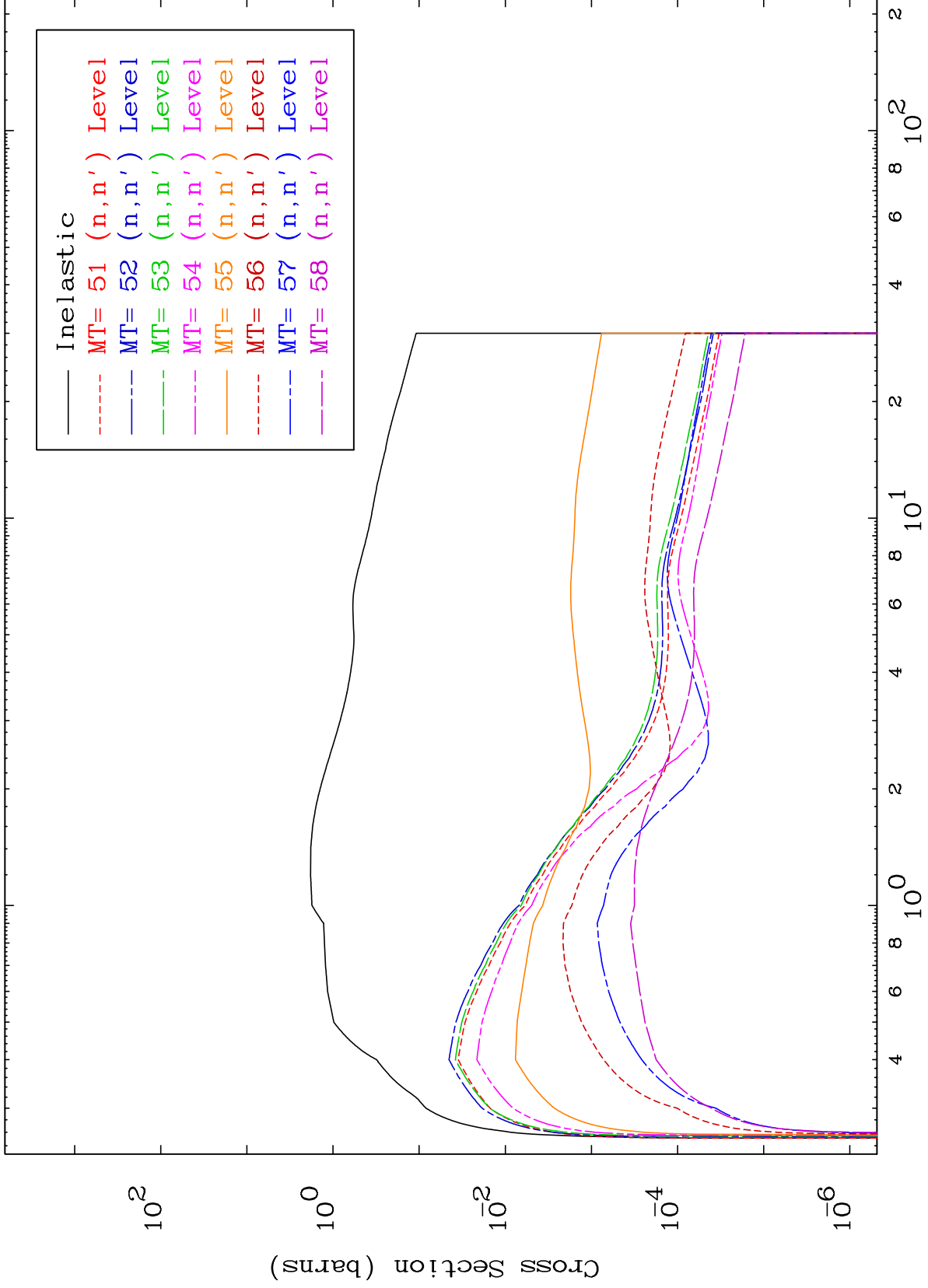


MAT 9990

(n,n') Levels

103-Lr-261

293 Kelvin Cross Sections

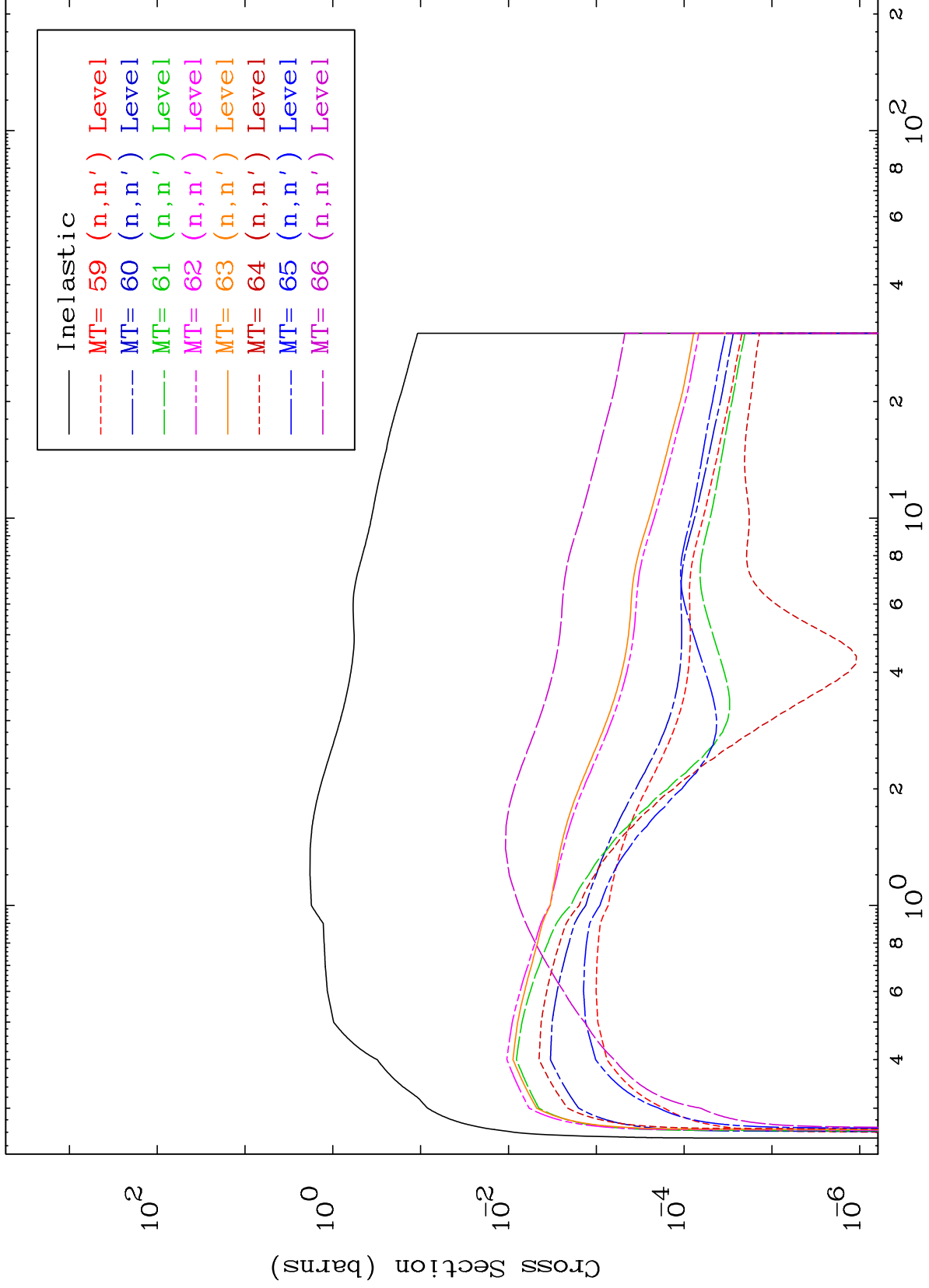


MAT 9990

(n,n') Levels

103-Lr-261

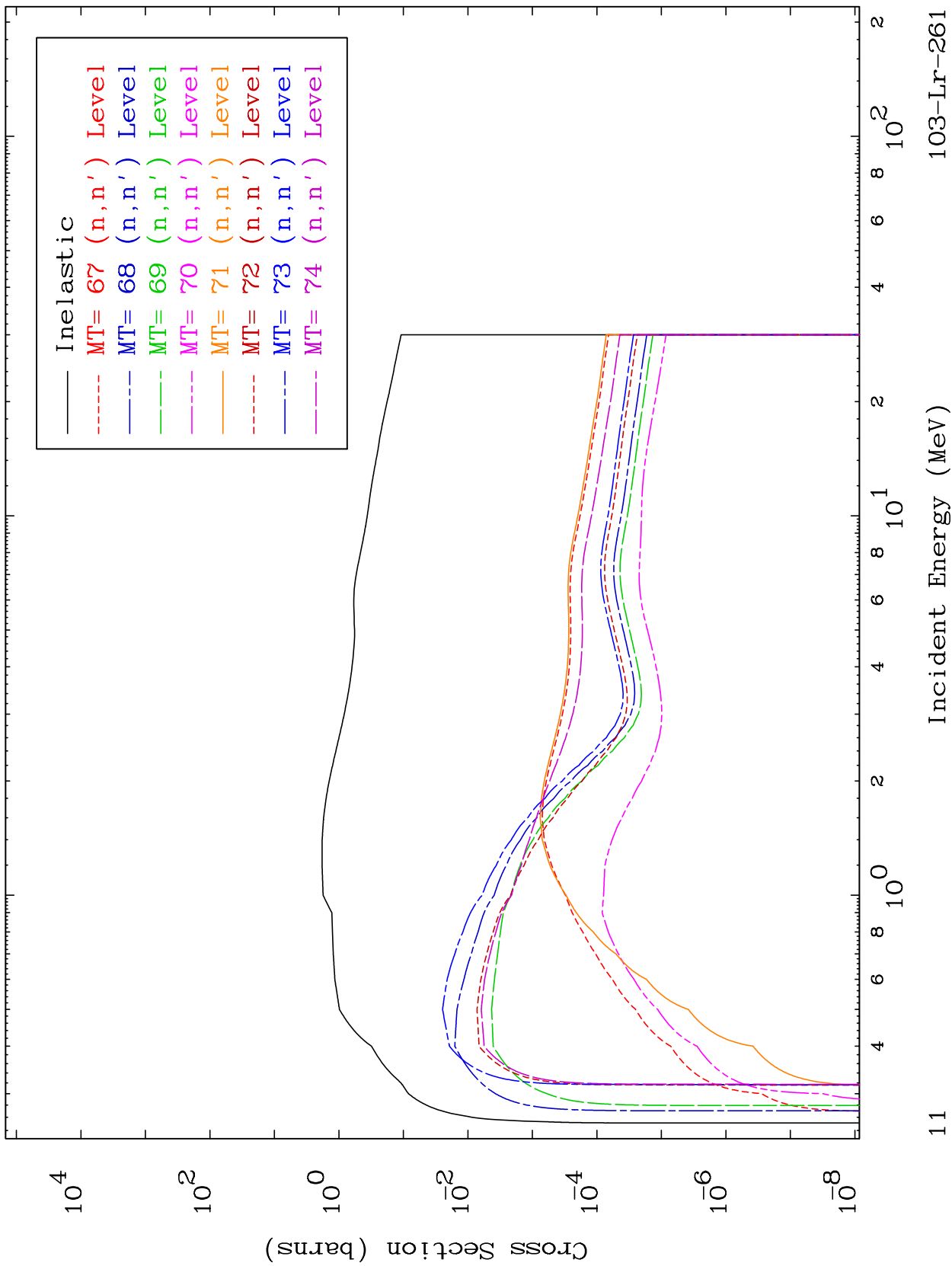
293 Kelvin Cross Sections



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Incident Energy (MeV)

103-Lr-261

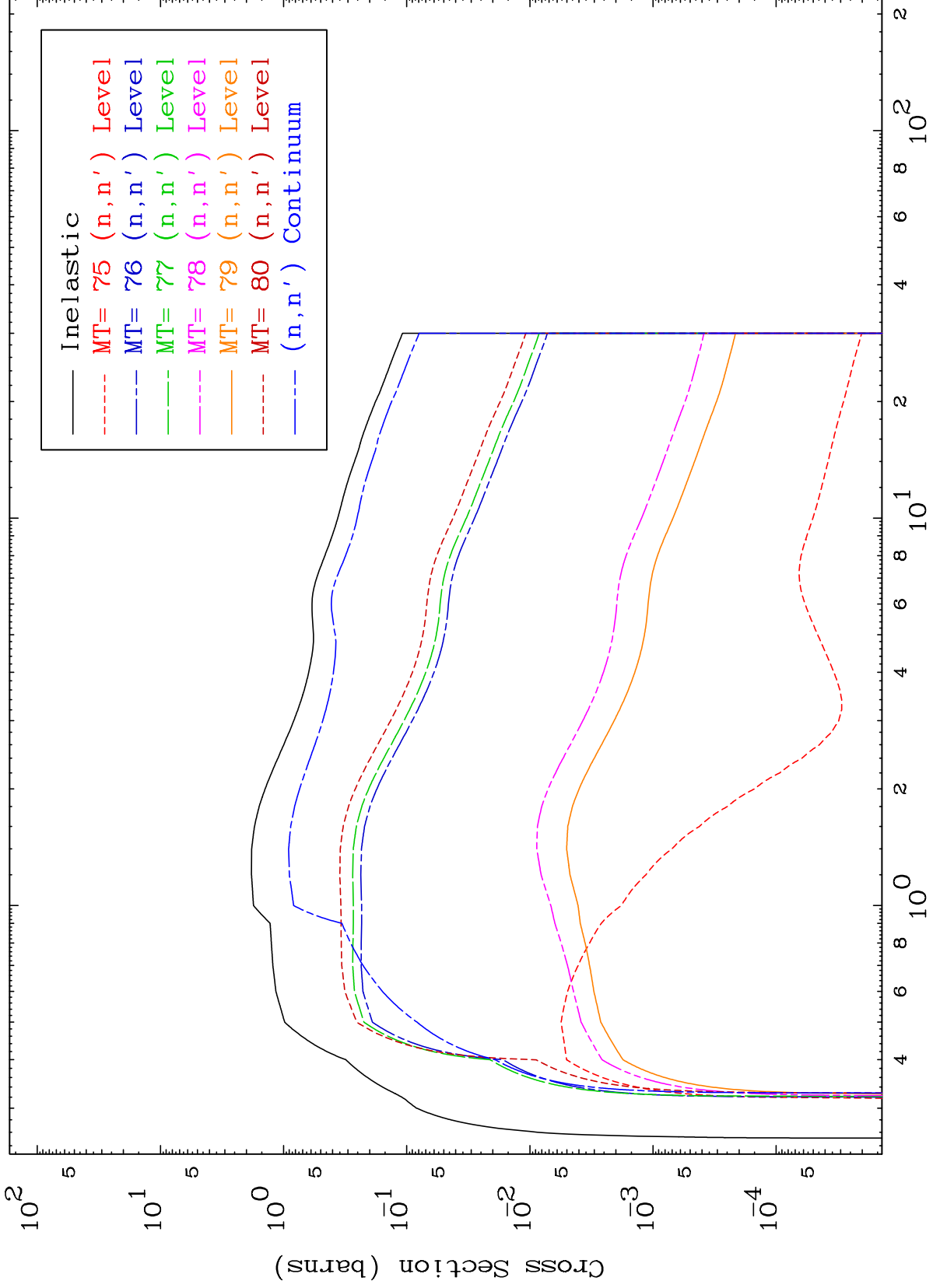


MAT 9990

(n,n') Levels

293 Kelvin Cross Sections

103-Lr-261



12

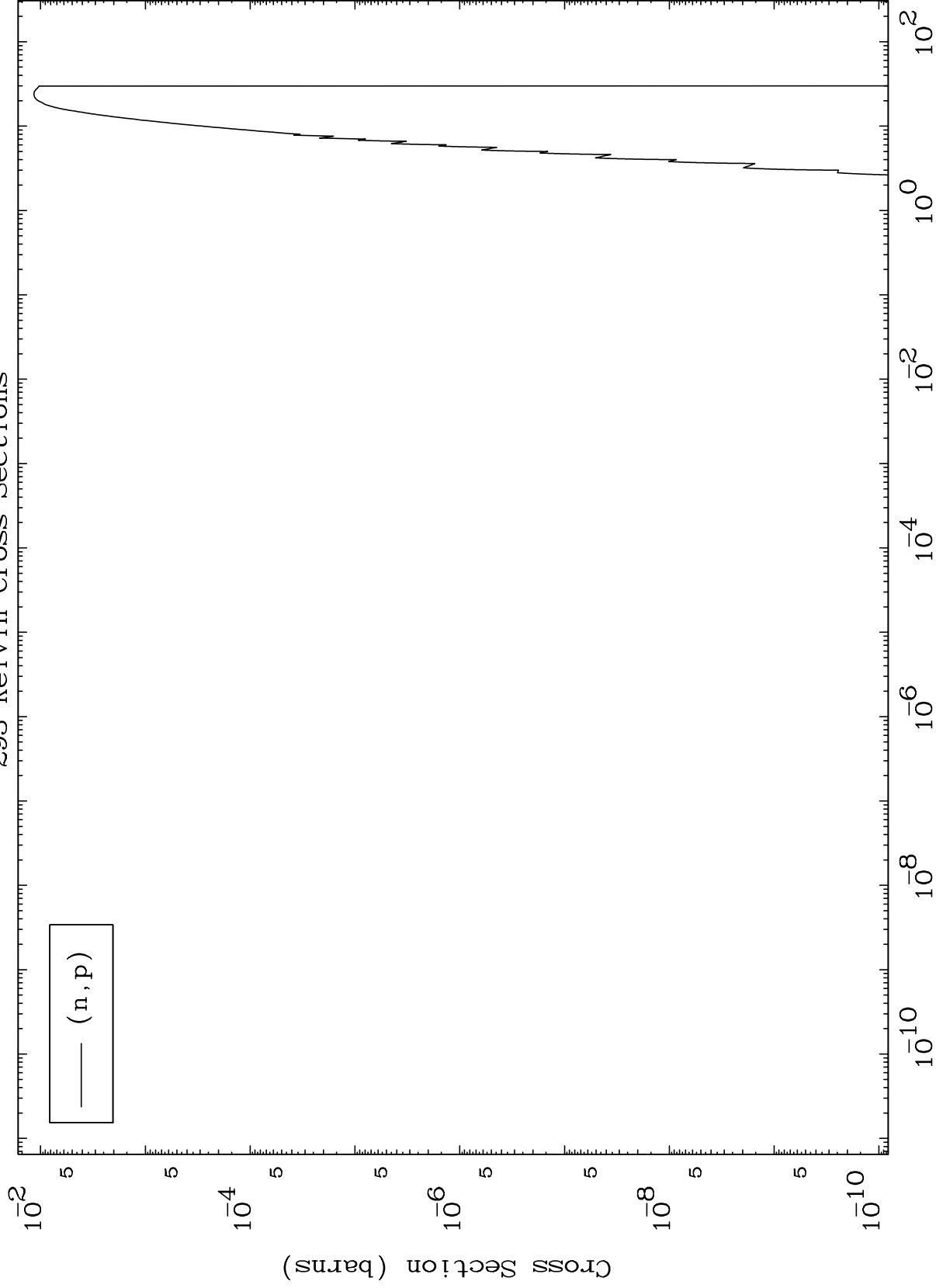
Incident Energy (MeV)

103-Lr-261

MAT 9990

(n,p) Levels
293 Kelvin Cross Sections

103-Lr-261



13

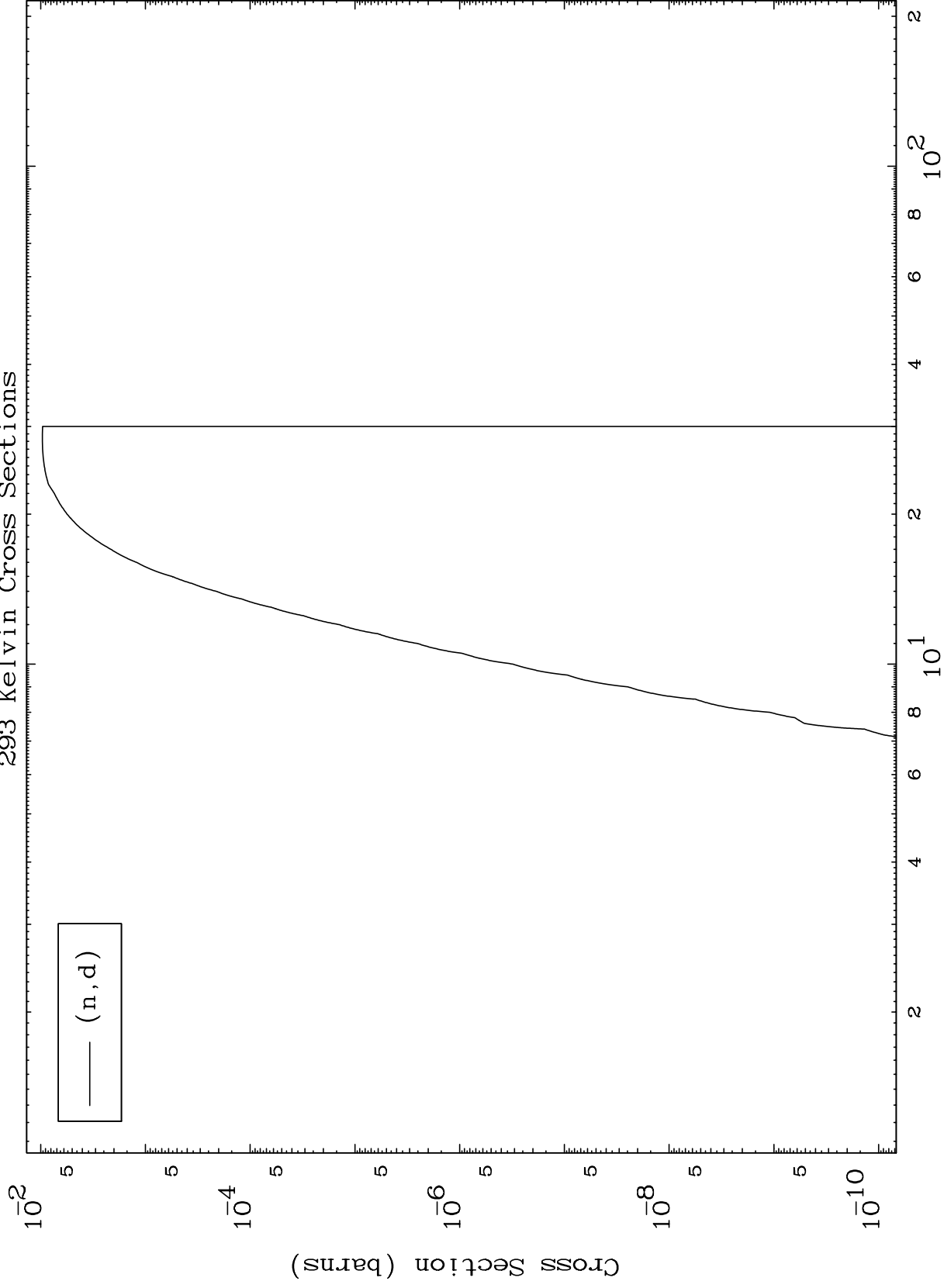
Incident Energy (MeV)

103-Lr-261

MAT 9990

(n,d) Levels
293 Kelvin Cross Sections

103-Lr-261



14

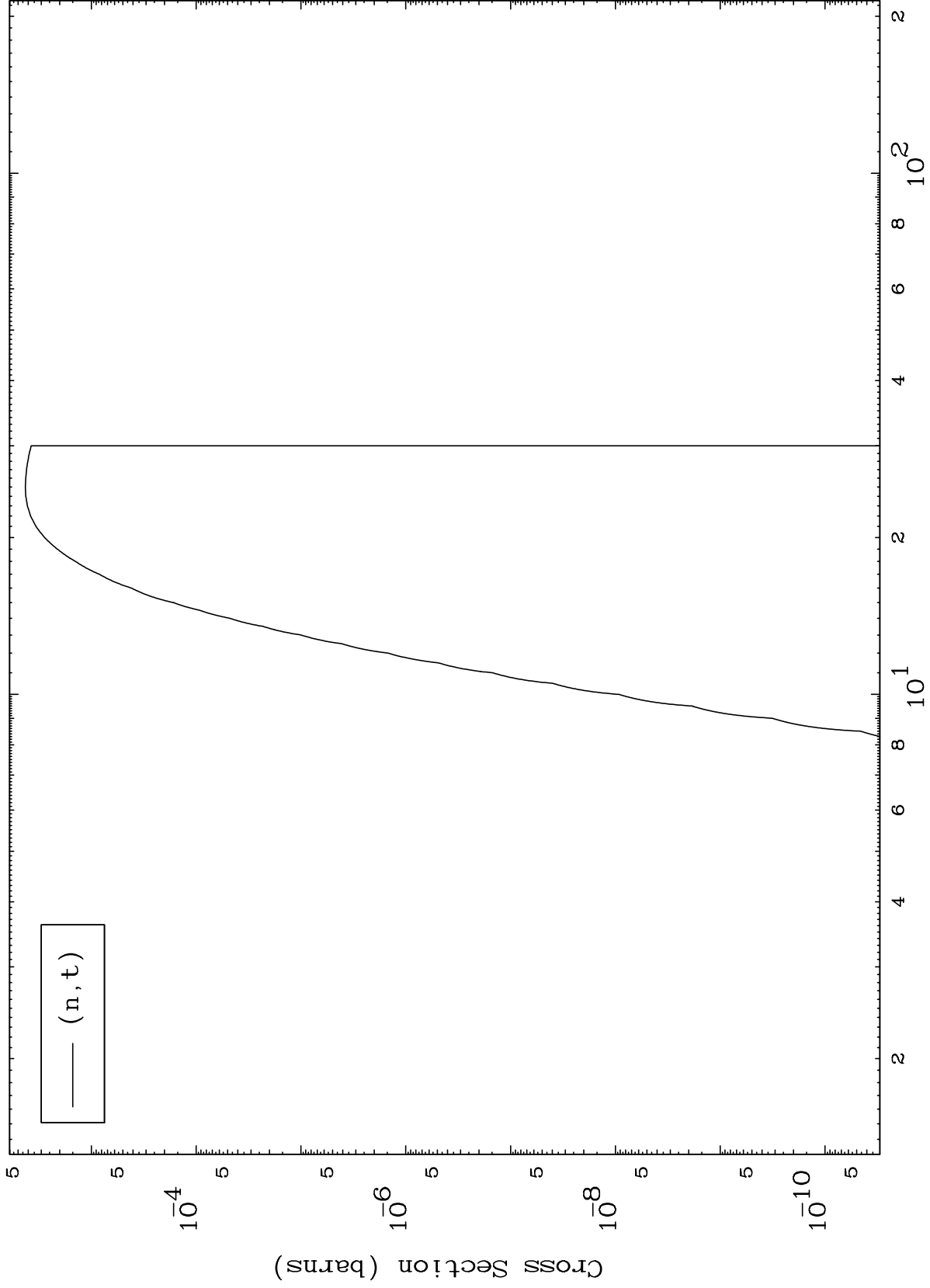
Incident Energy (MeV)

103-Lr-261

MAT 9990

(n,t) Levels
293 Kelvin Cross Sections

103-Lr-261



15

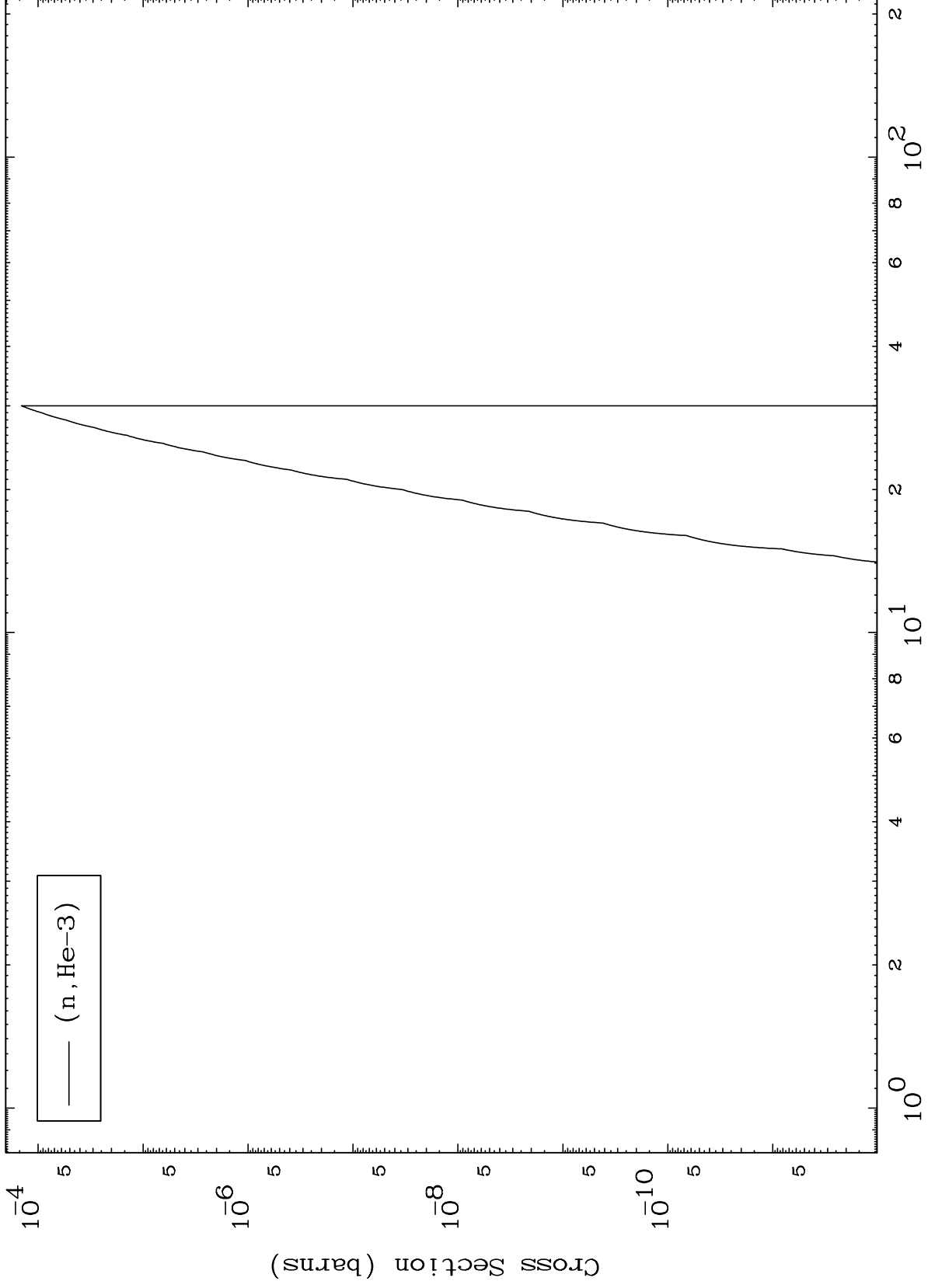
Incident Energy (MeV)

103-Lr-261

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(n,He3) Levels
293 Kelvin Cross Sections

103-Lr-261



16

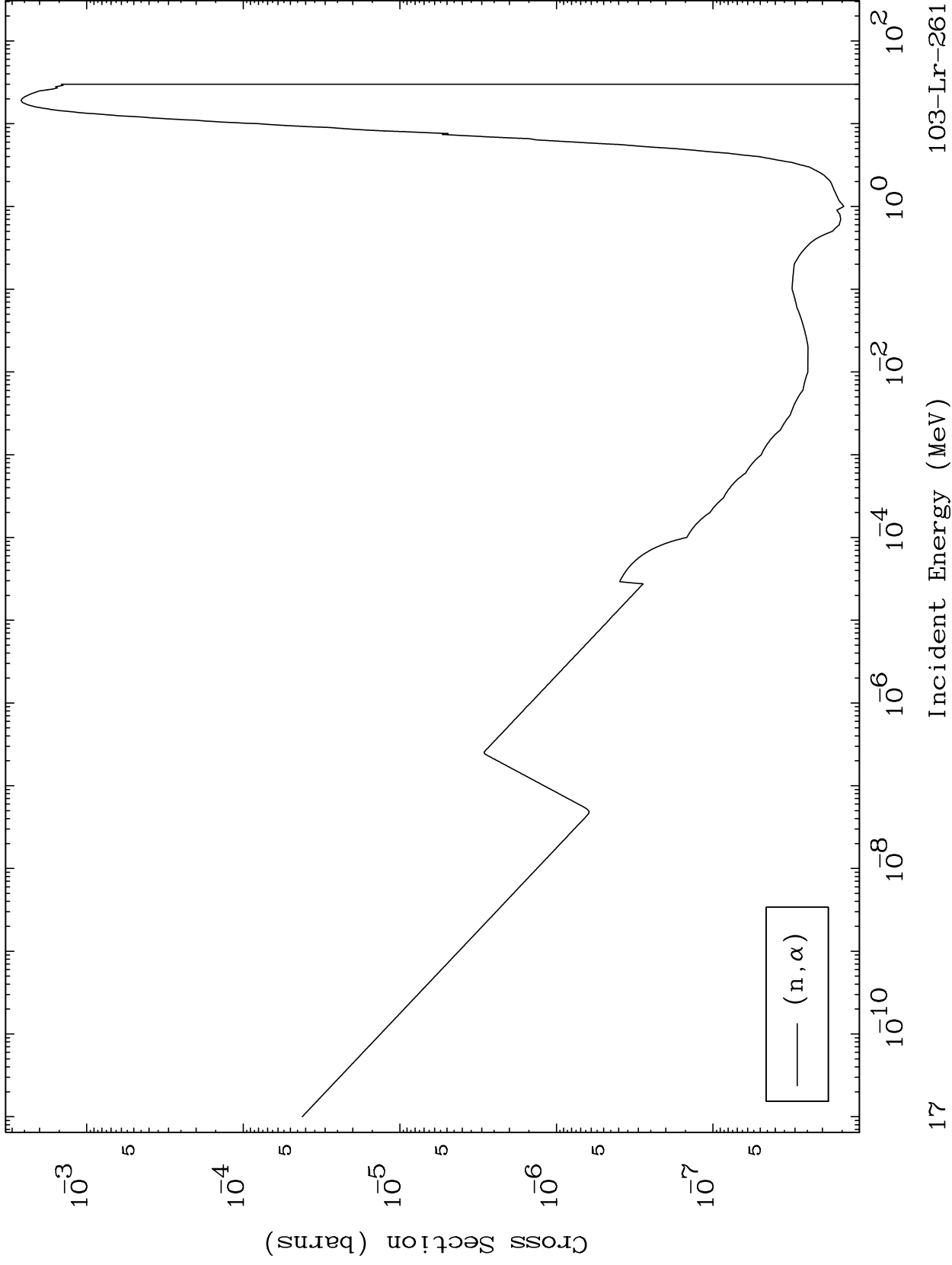
Incident Energy (MeV)

103-Lr-261

MAT 9990

(n, α) Levels
293 Kelvin Cross Sections

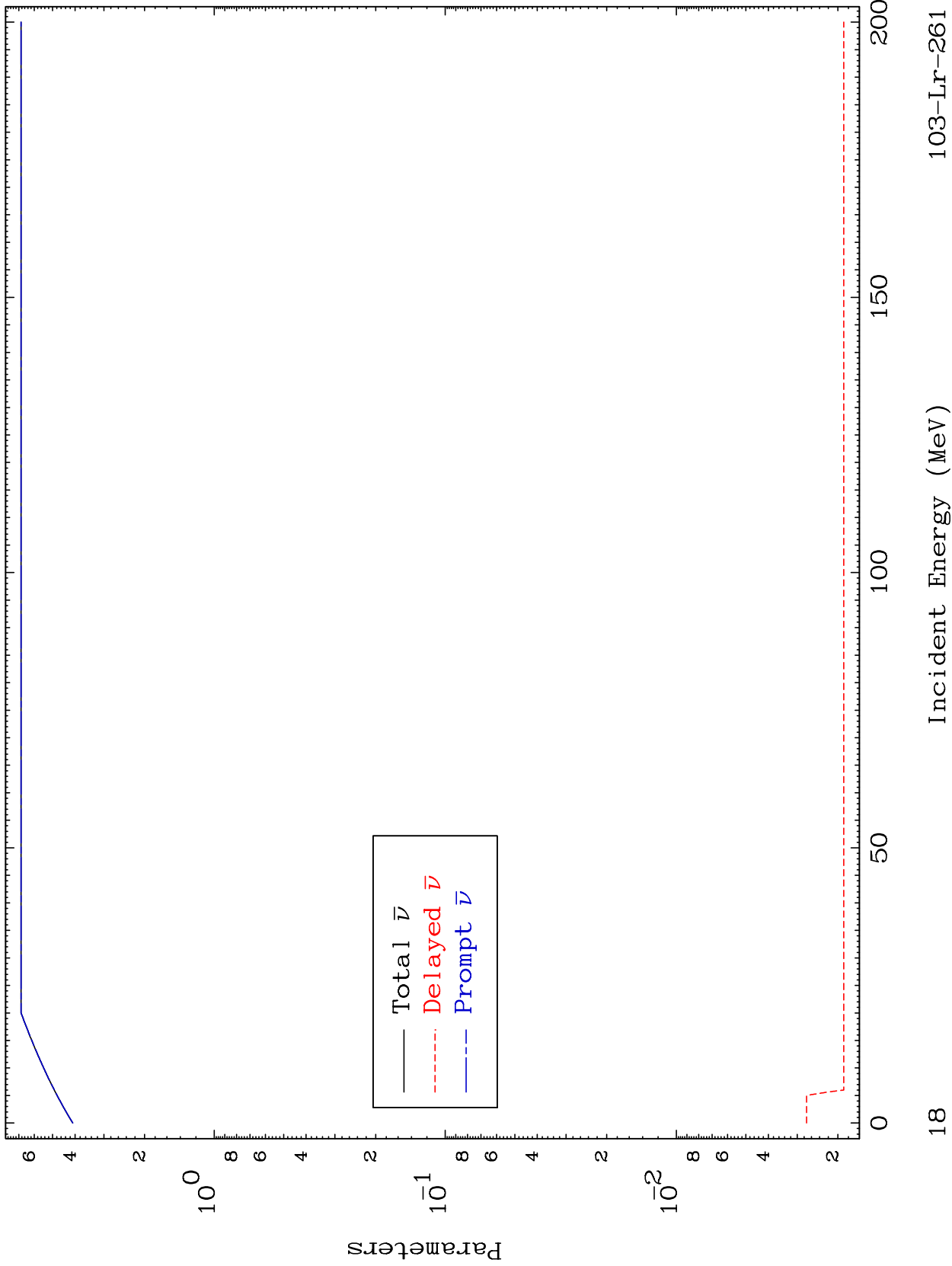
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Energy Release
Parameters

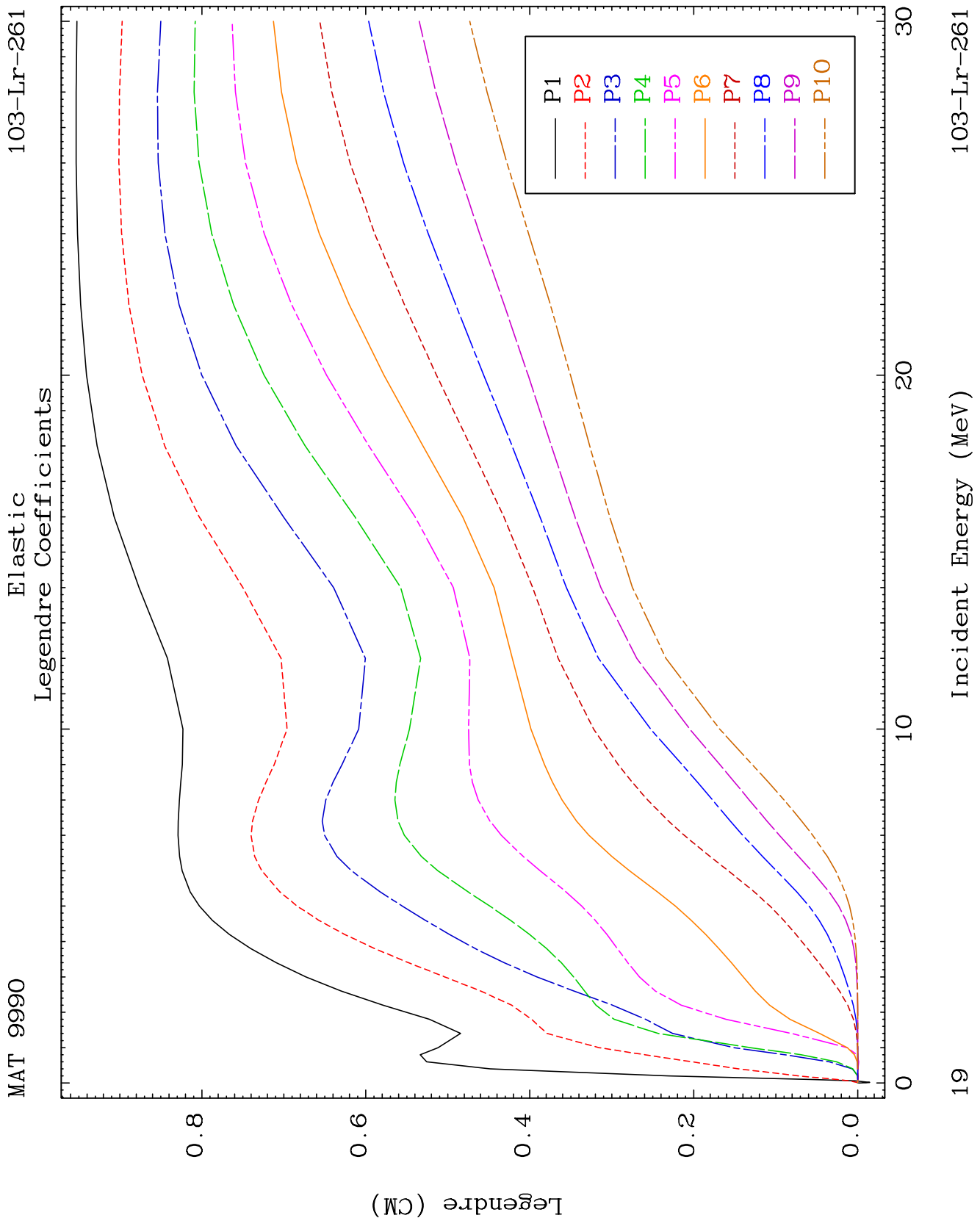
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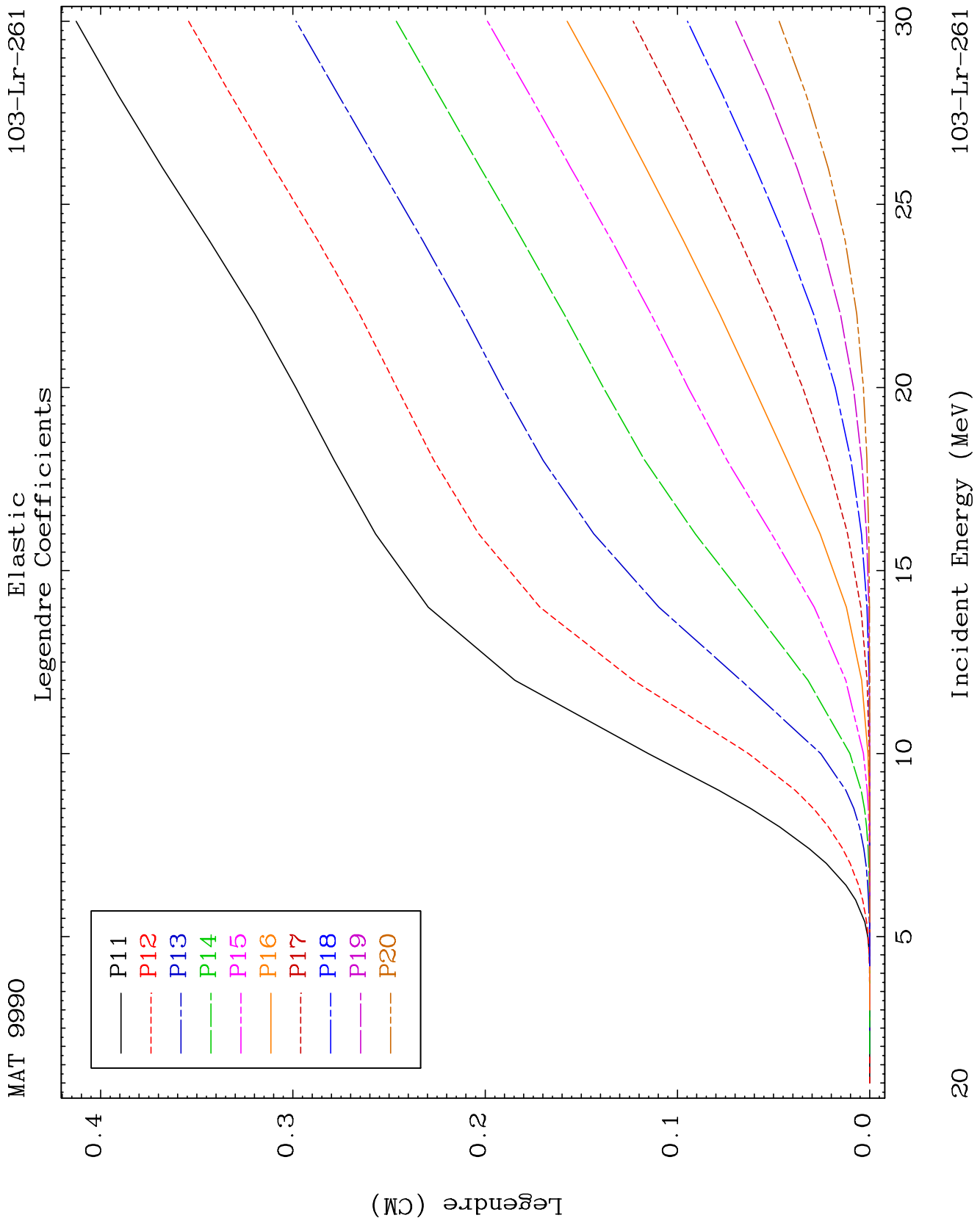


18

Incident Energy (MeV)

103-Lr-261

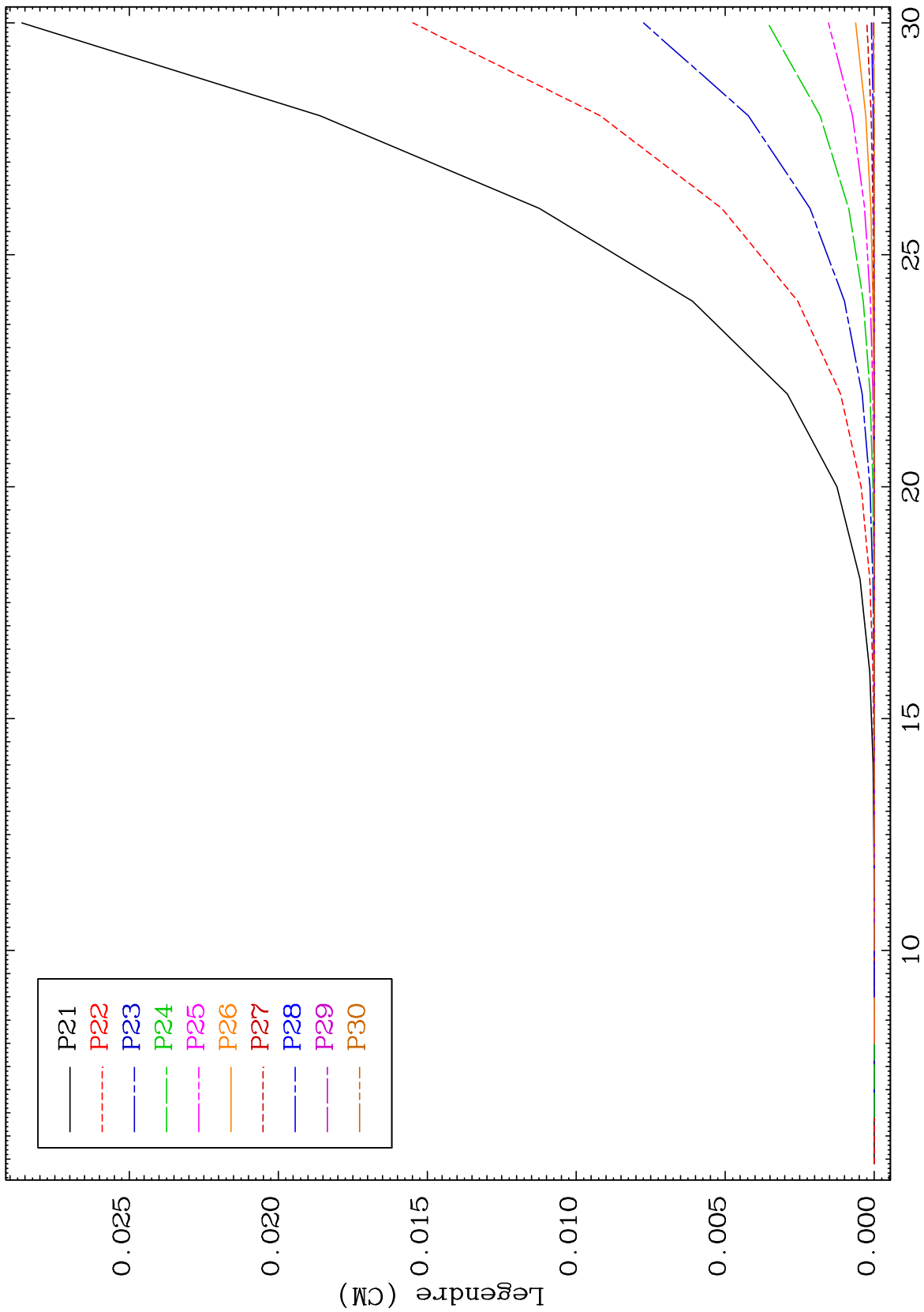




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Elastic Legendre Coefficients

103-Lr-261



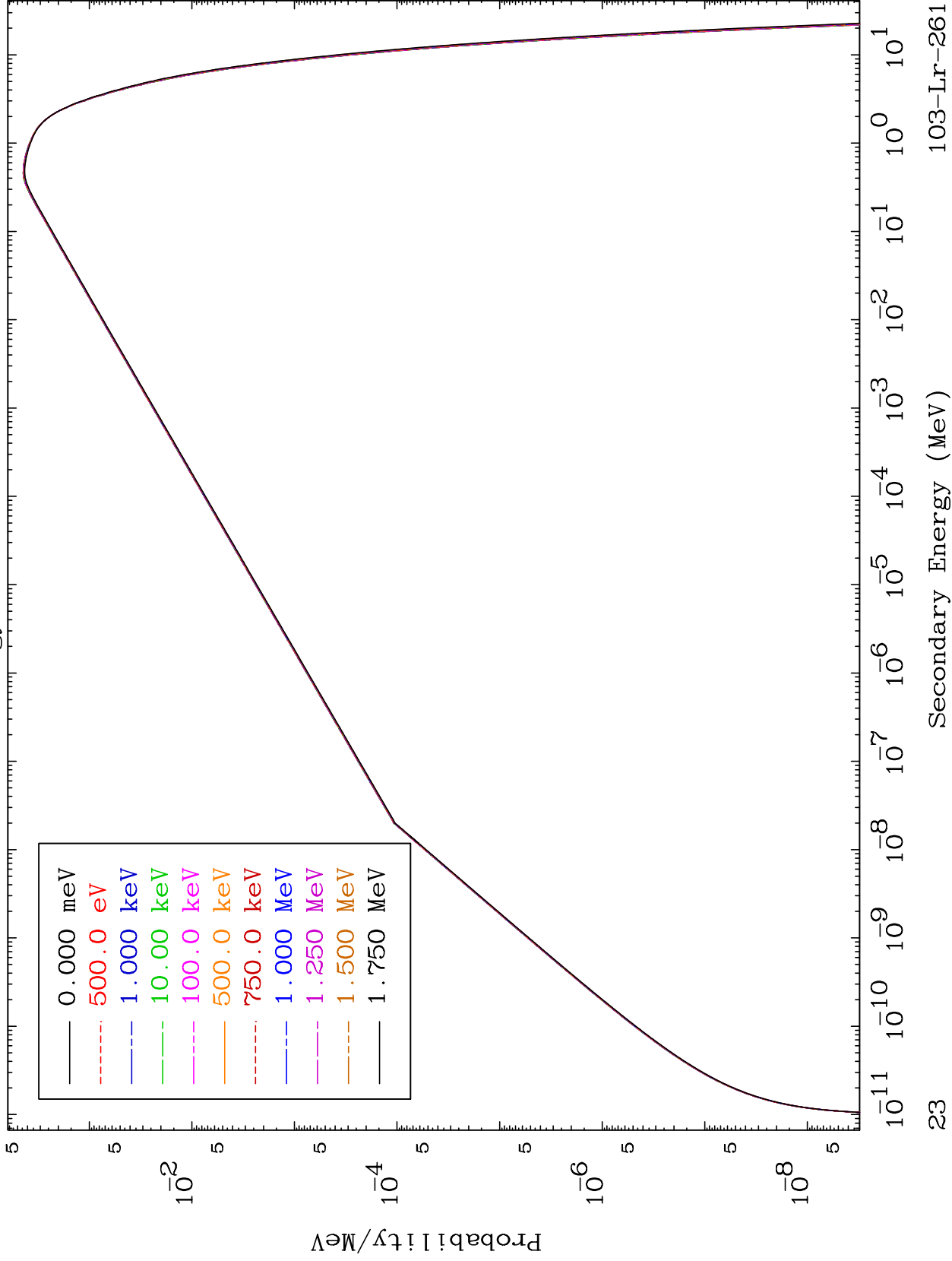
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Incident Energy (MeV)

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Fission Energy Distributions

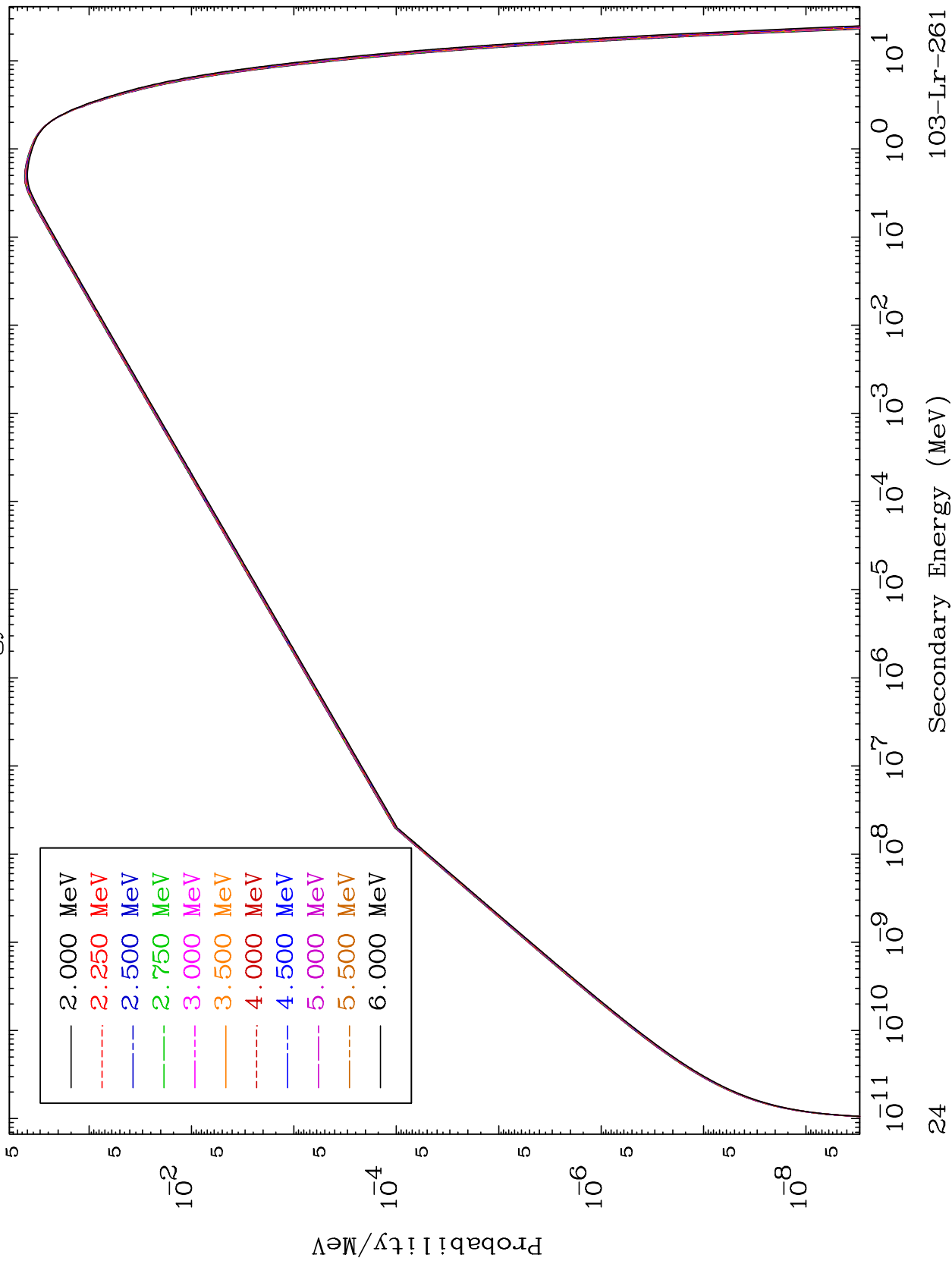
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Fission Energy Distributions

103-Lr-261



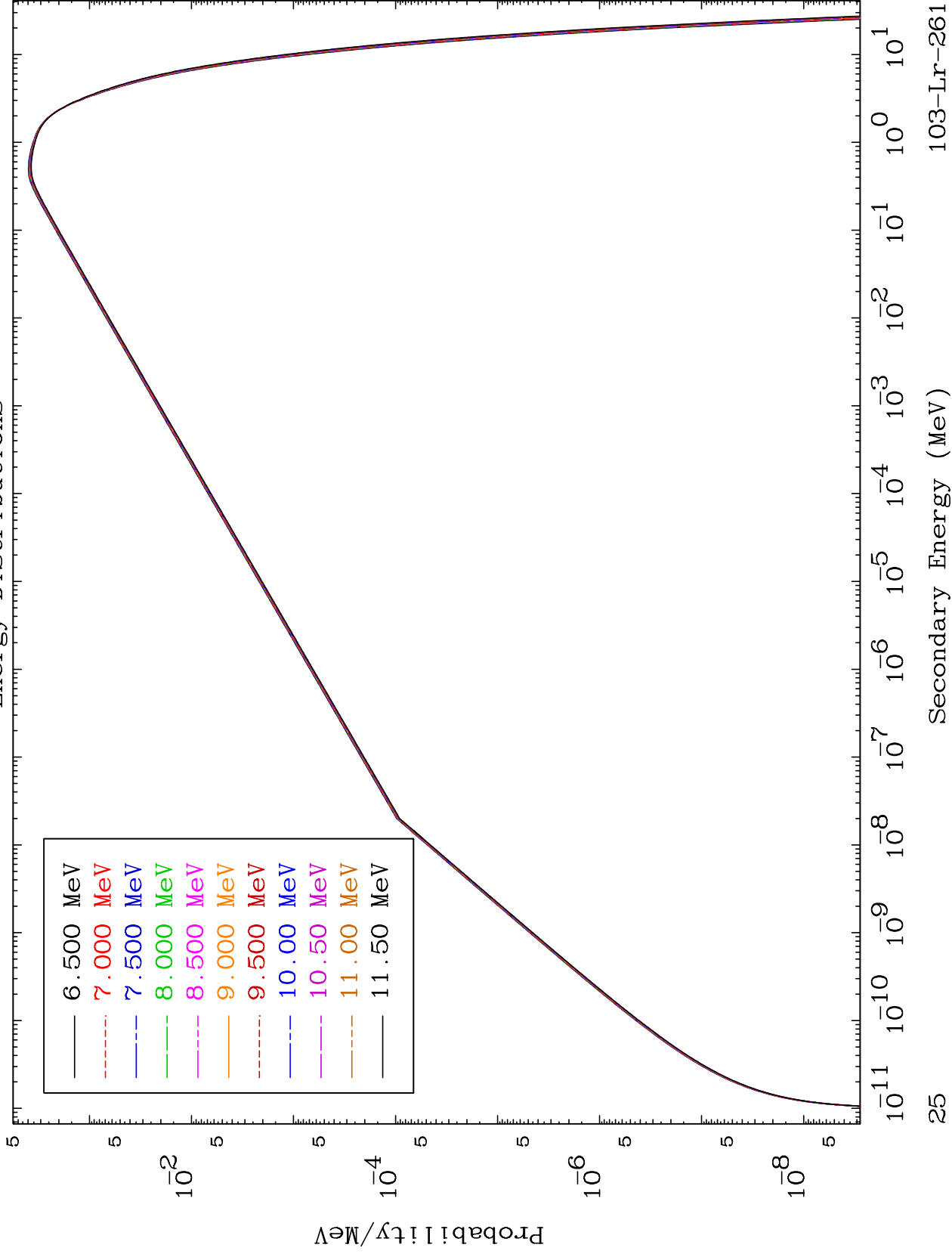
103-Lr-261

Secondary Energy (MeV)

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Fission Energy Distributions

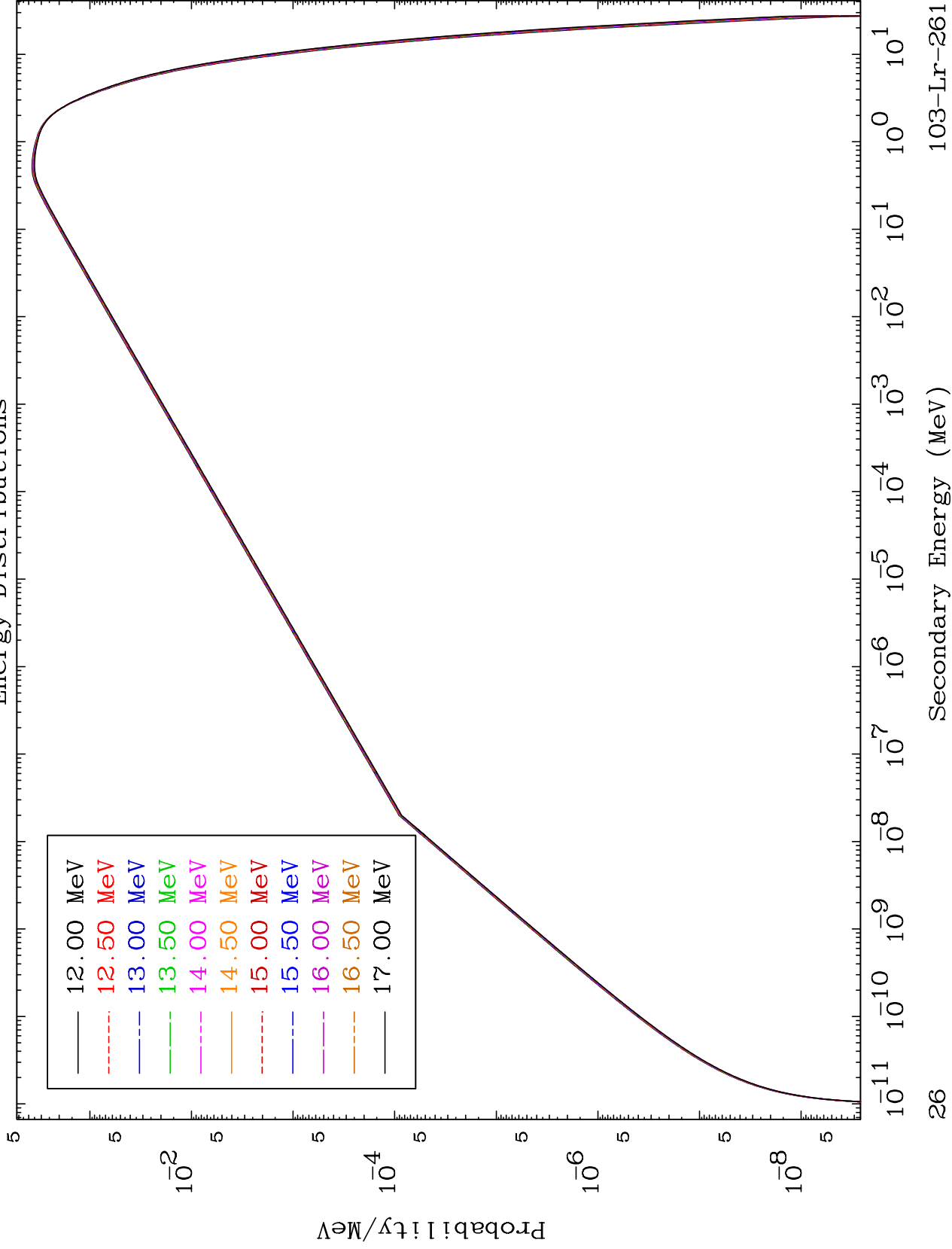
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Fission Energy Distributions

103-Lr-261



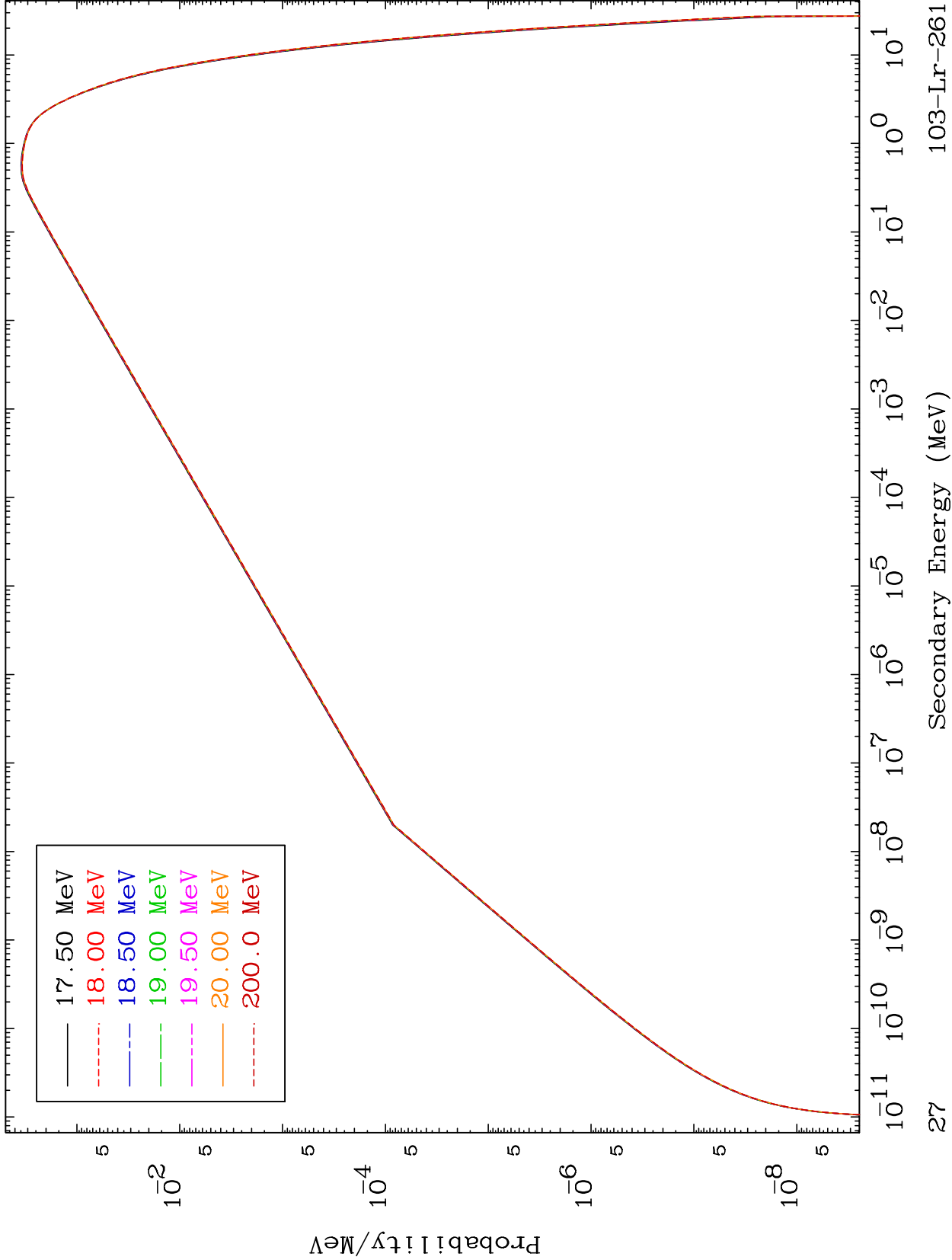
26

103-Lr-261

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Fission Energy Distributions

103-Lr-261

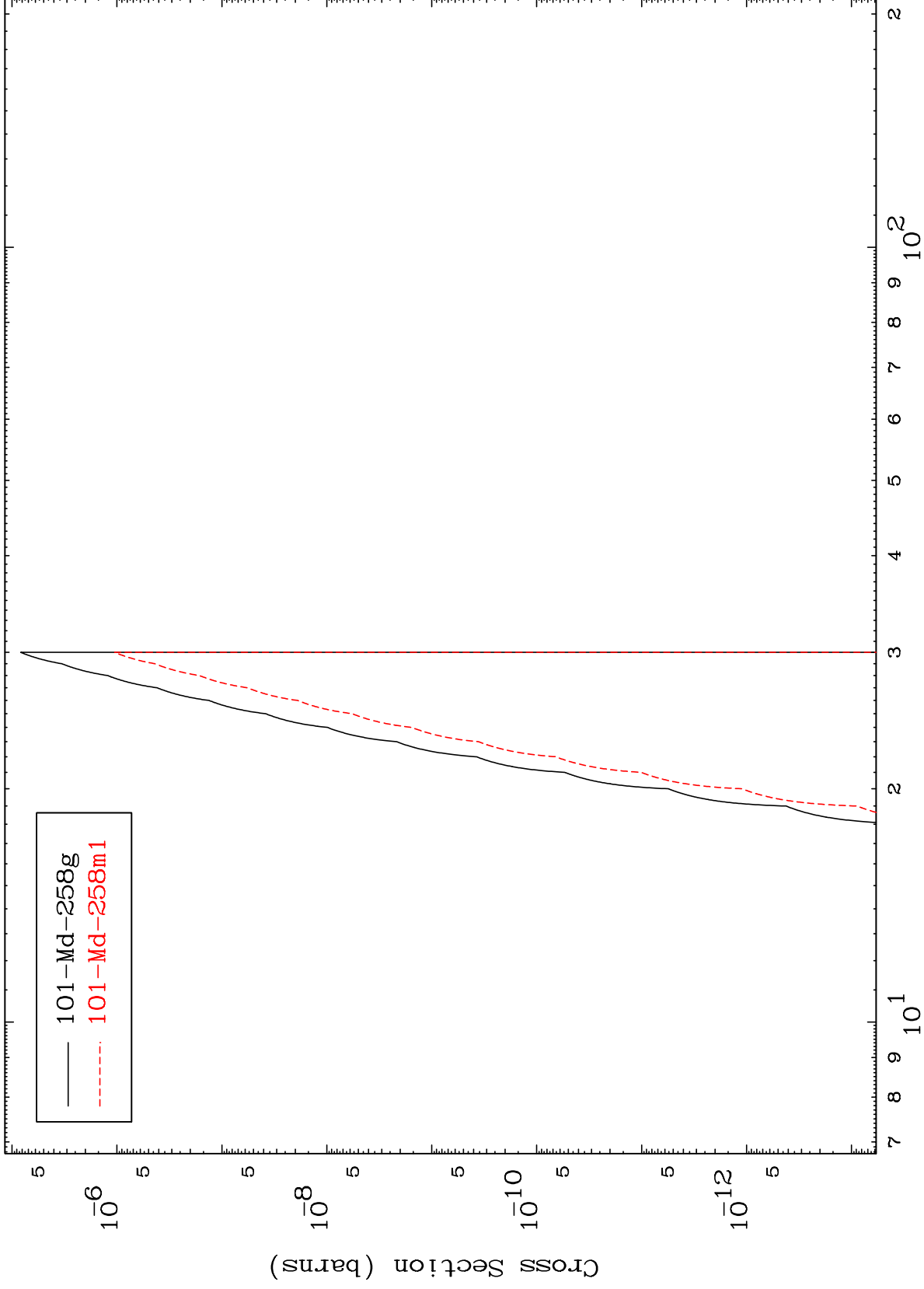


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(n,n') He-3

103-Lr-261

Radionuclide Production Cross Section



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Incident Energy (MeV)

103-Lr-261

MAT 9990

(n, α)

103-Lr-261

Radionuclide Production Cross Section

