

Program EVALPLOT
(Version 2021-1)

by

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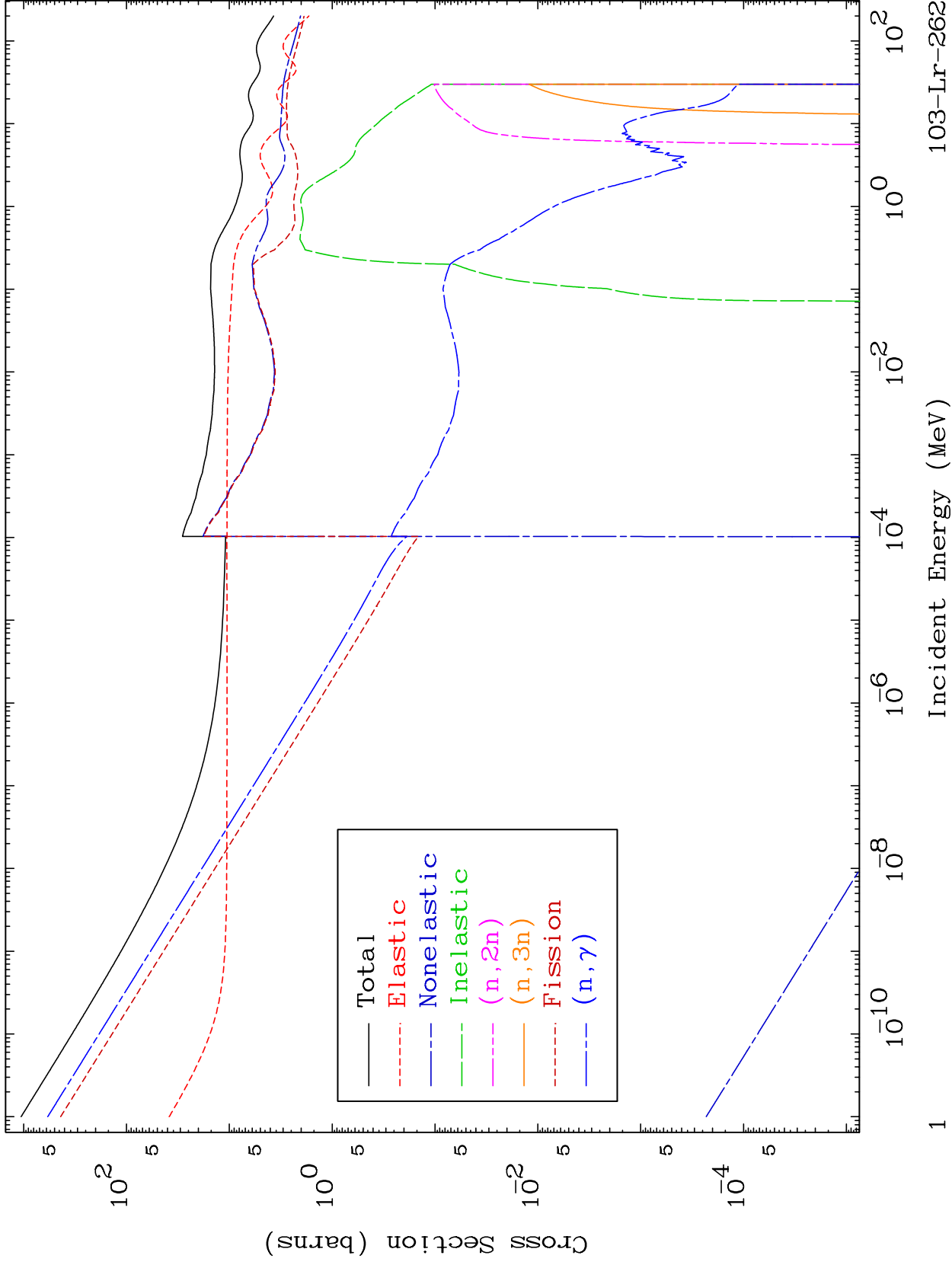
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Press Mouse Button to Start

MAT 9991

Neutron Major
293 Kelvin Cross Sections

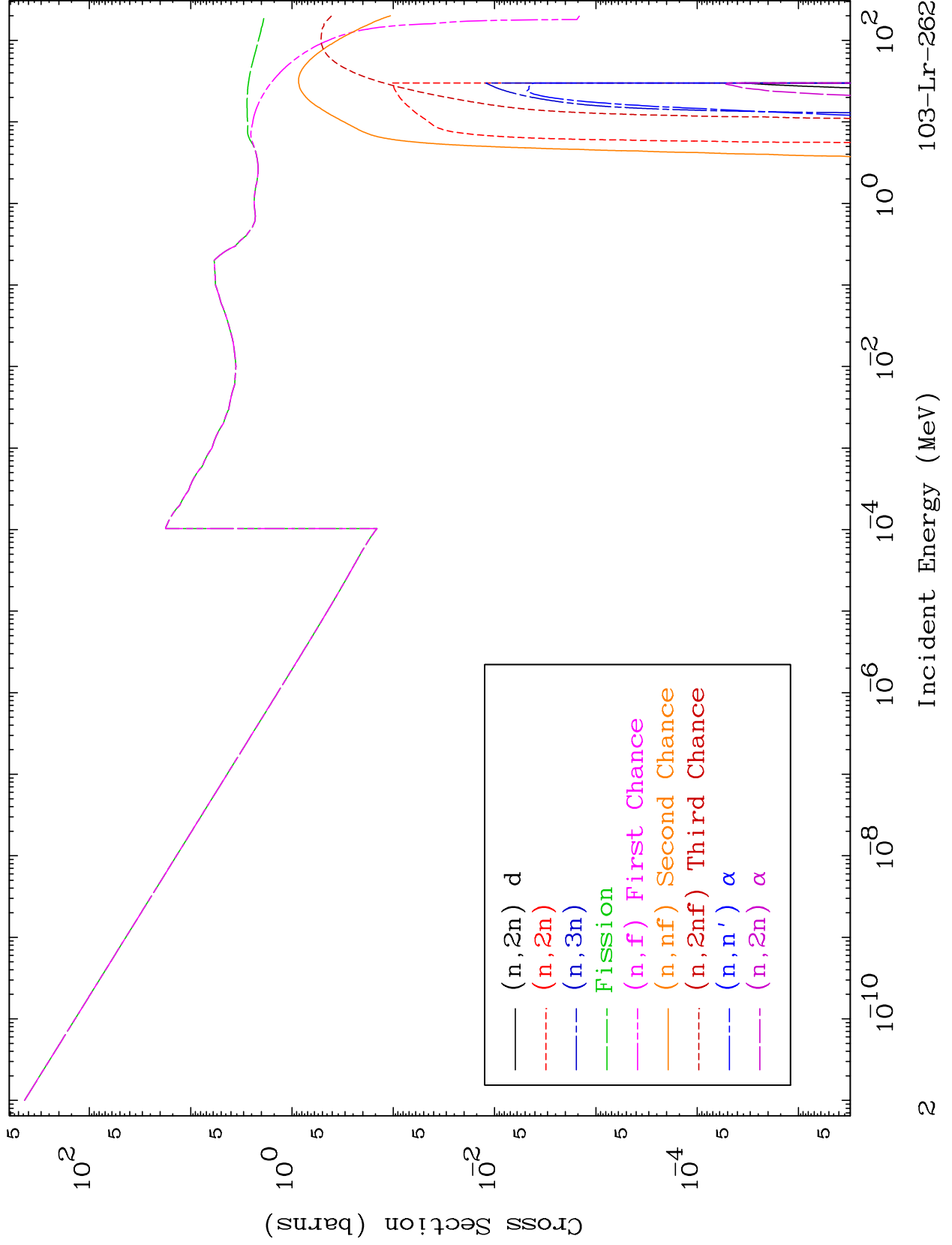
103-Lr-262

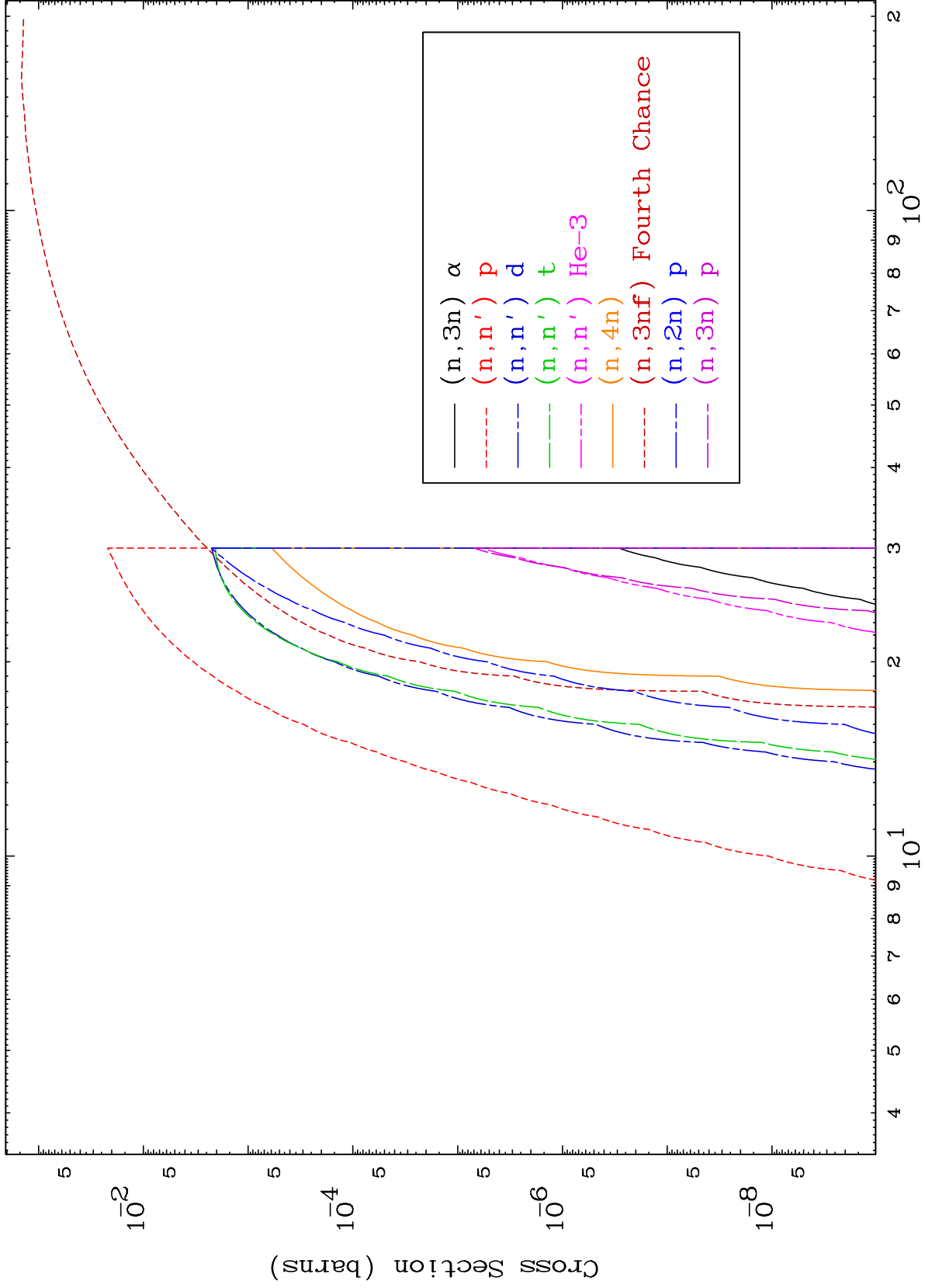


MAT 9991

Neutron Absorption
293 Kelvin Cross Sections

103-Lr-262

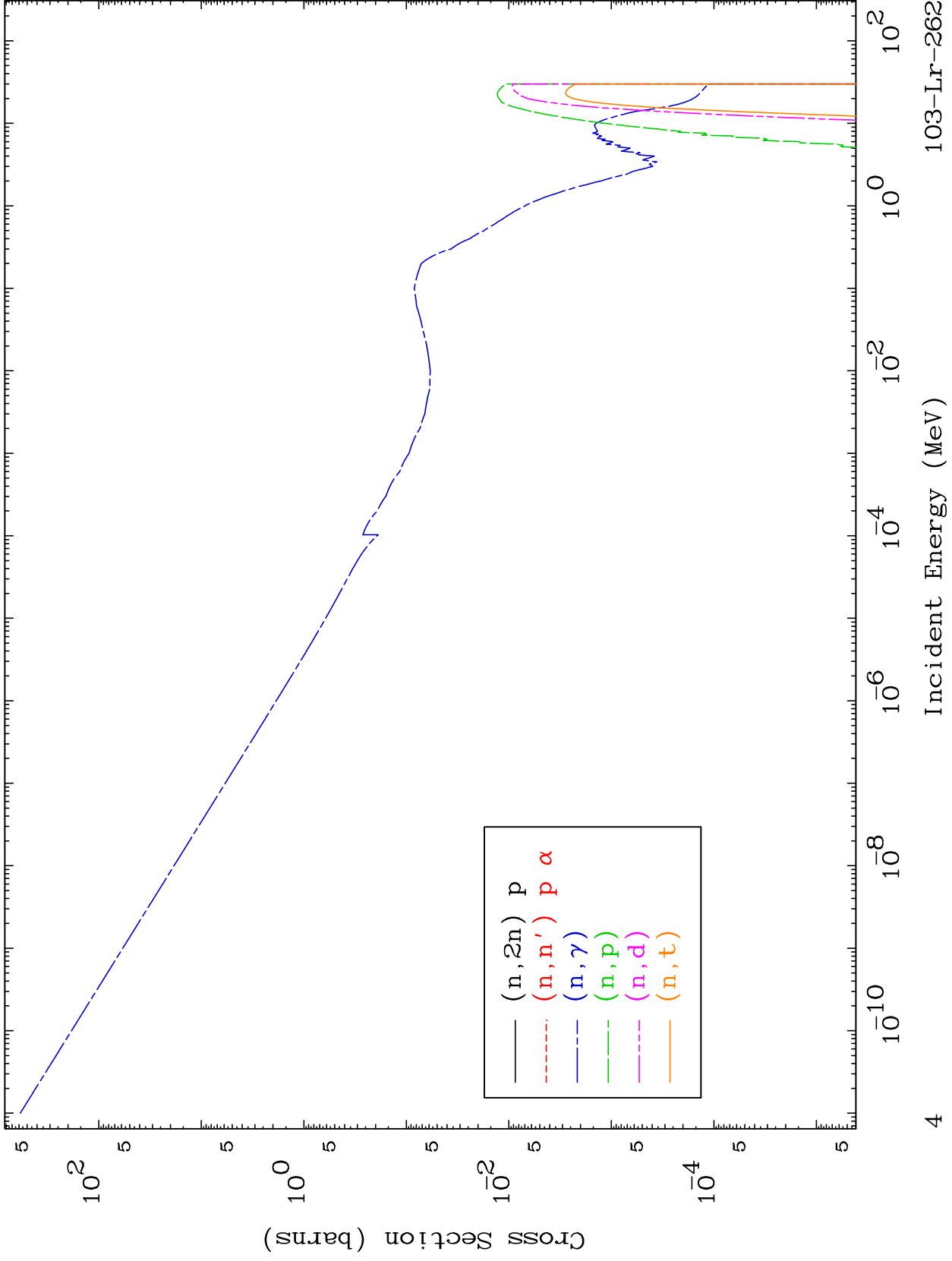


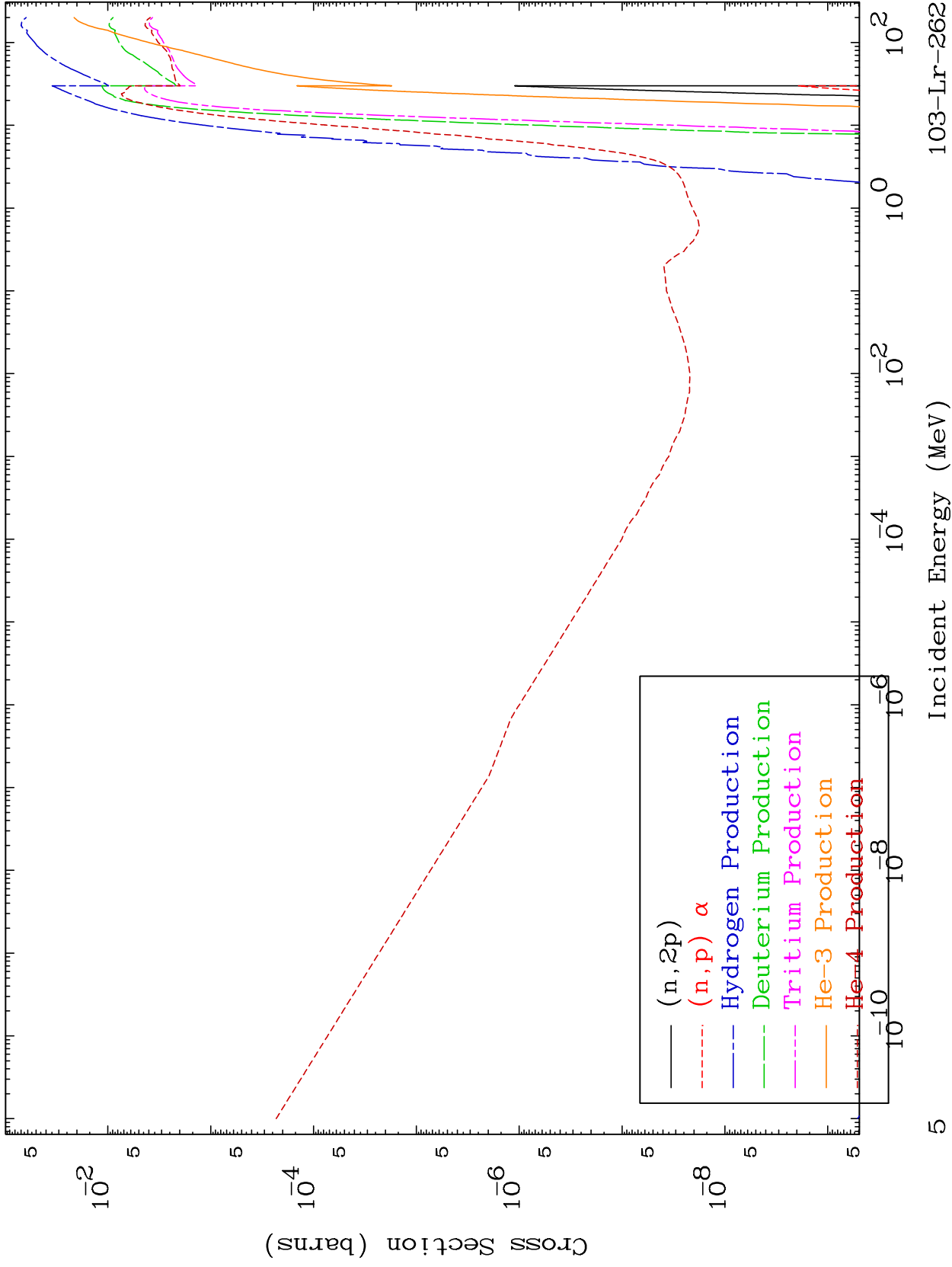


MAT 9991

Neutron Absorption
293 Kelvin Cross Sections

103-Lr-262

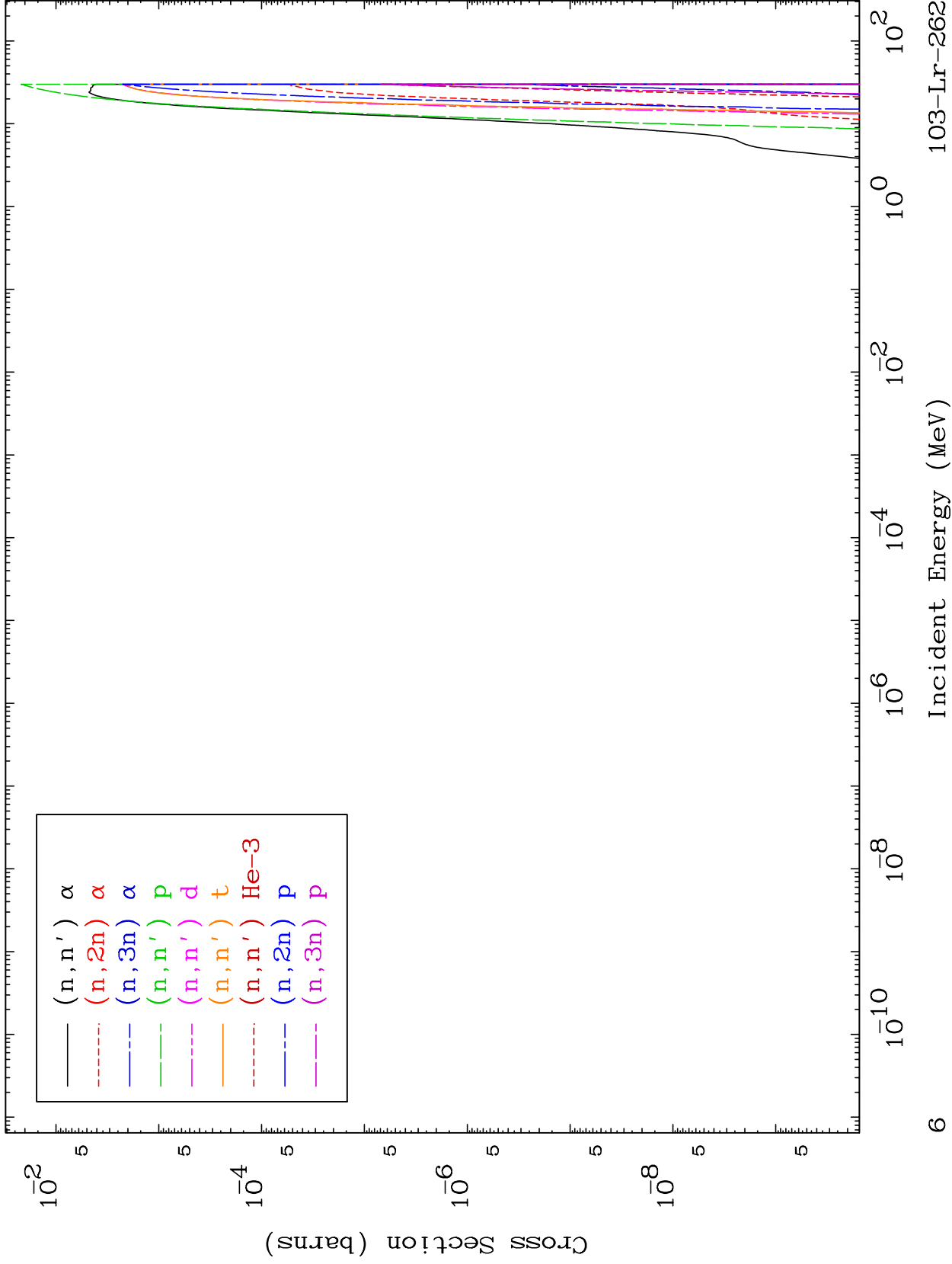




MAT 9991

Charged Particle
293 Kelvin Cross Sections

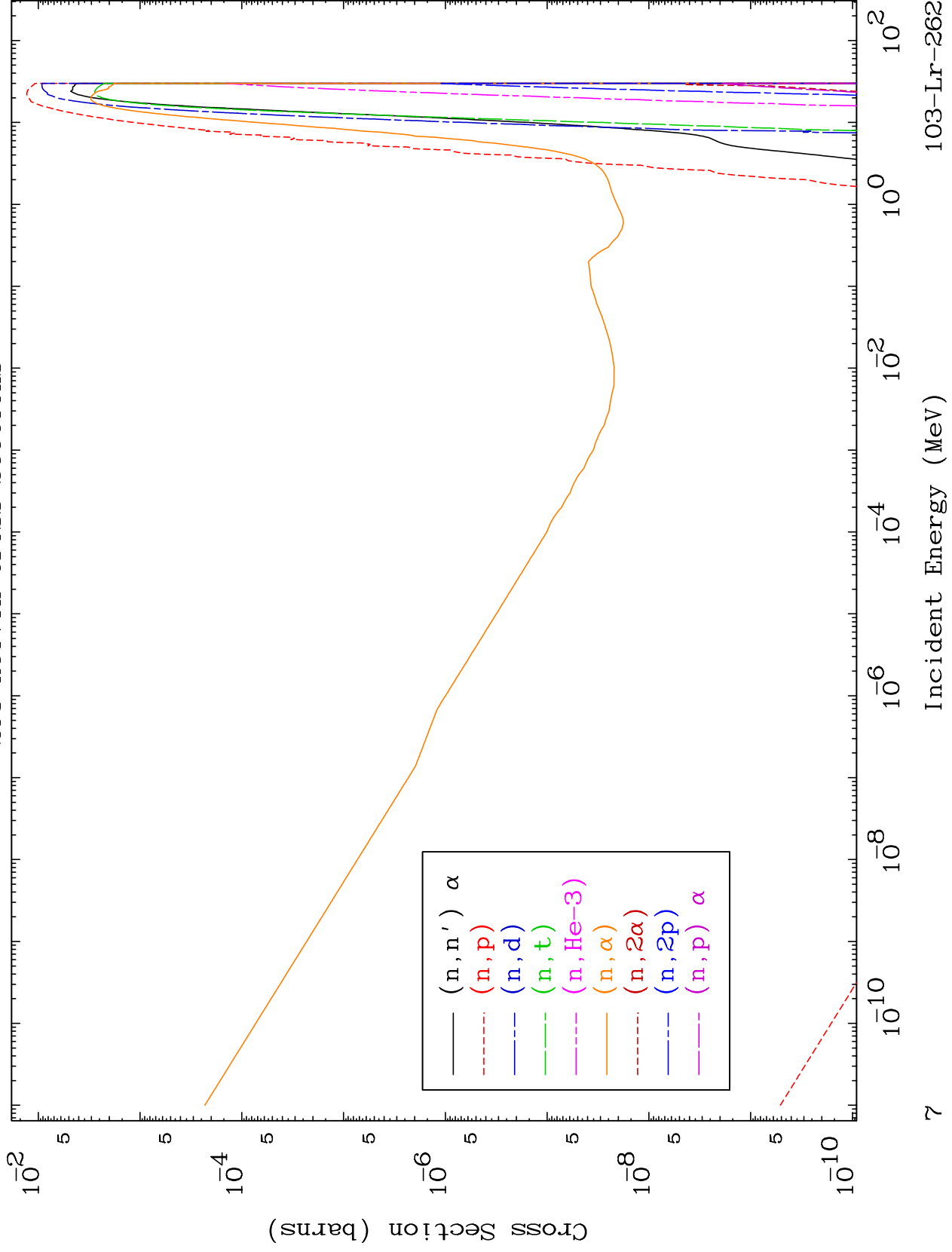
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Charged Particle
293 Kelvin Cross Sections

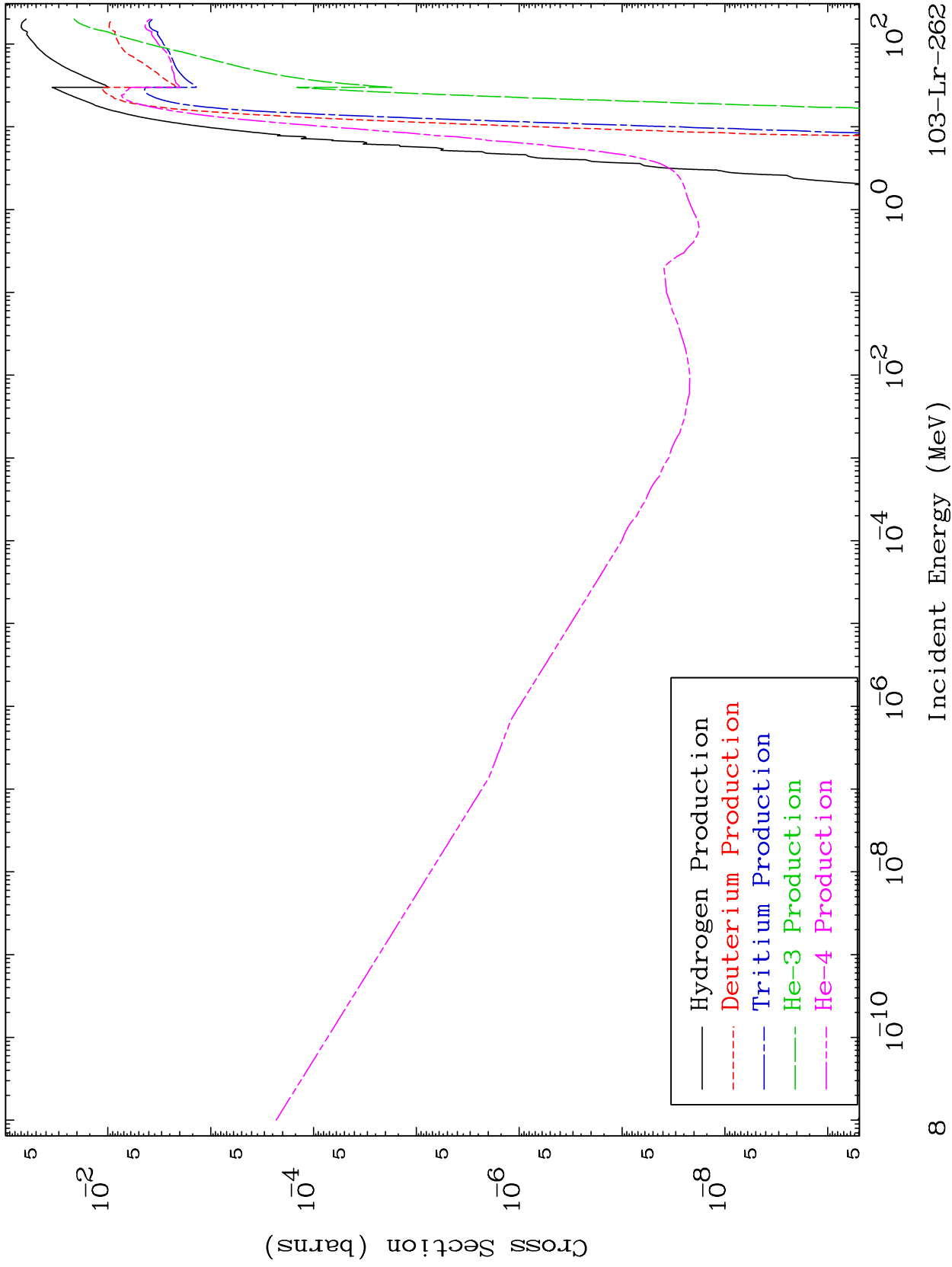
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Particle Production
293 Kelvin Cross Sections

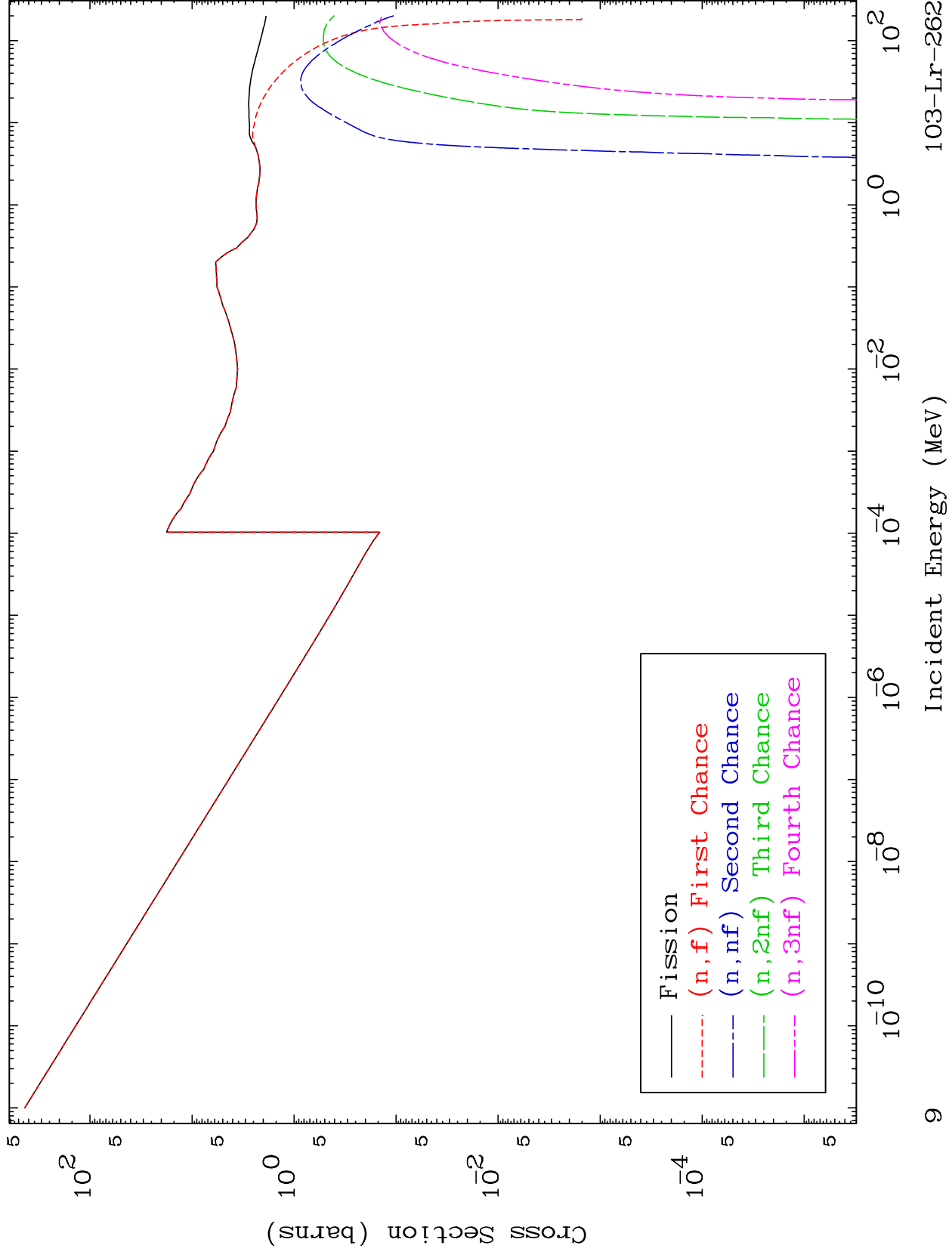
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Fission
293 Kelvin Cross Sections

103-Lr-262

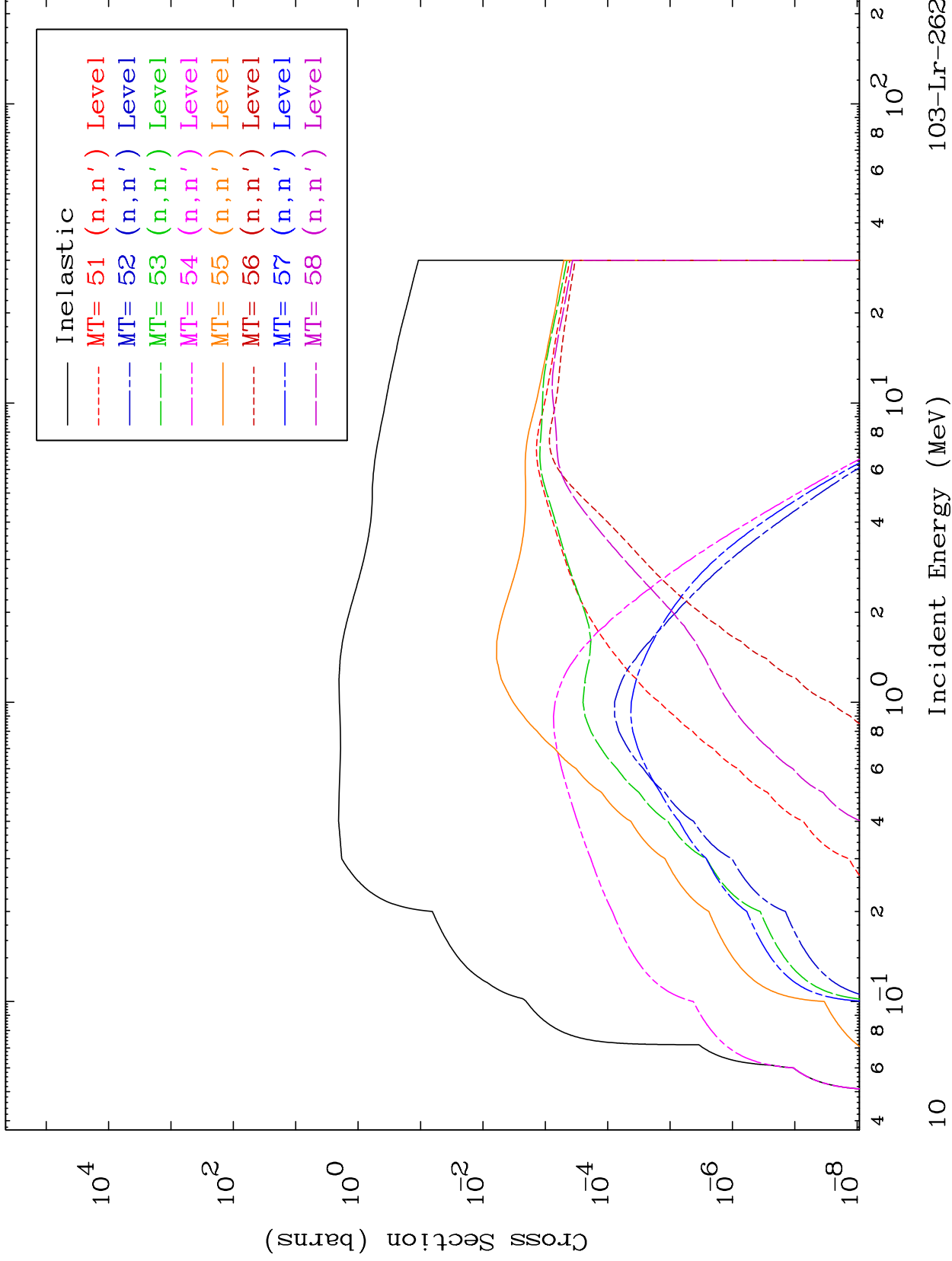


MAT 9991

(n,n') Levels

103-Lr-262

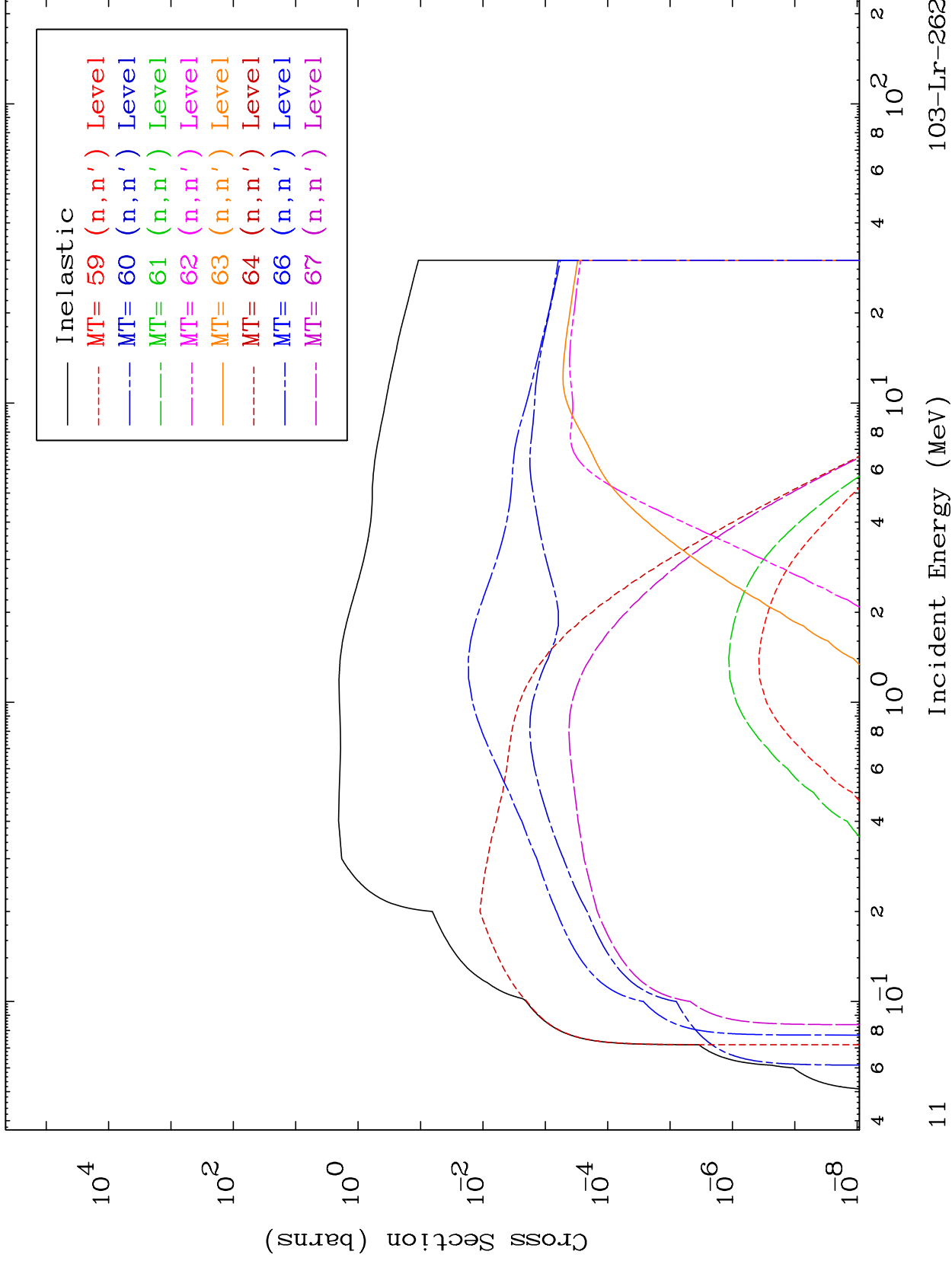
293 Kelvin Cross Sections



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Incident Energy (MeV)

103-Lr-262

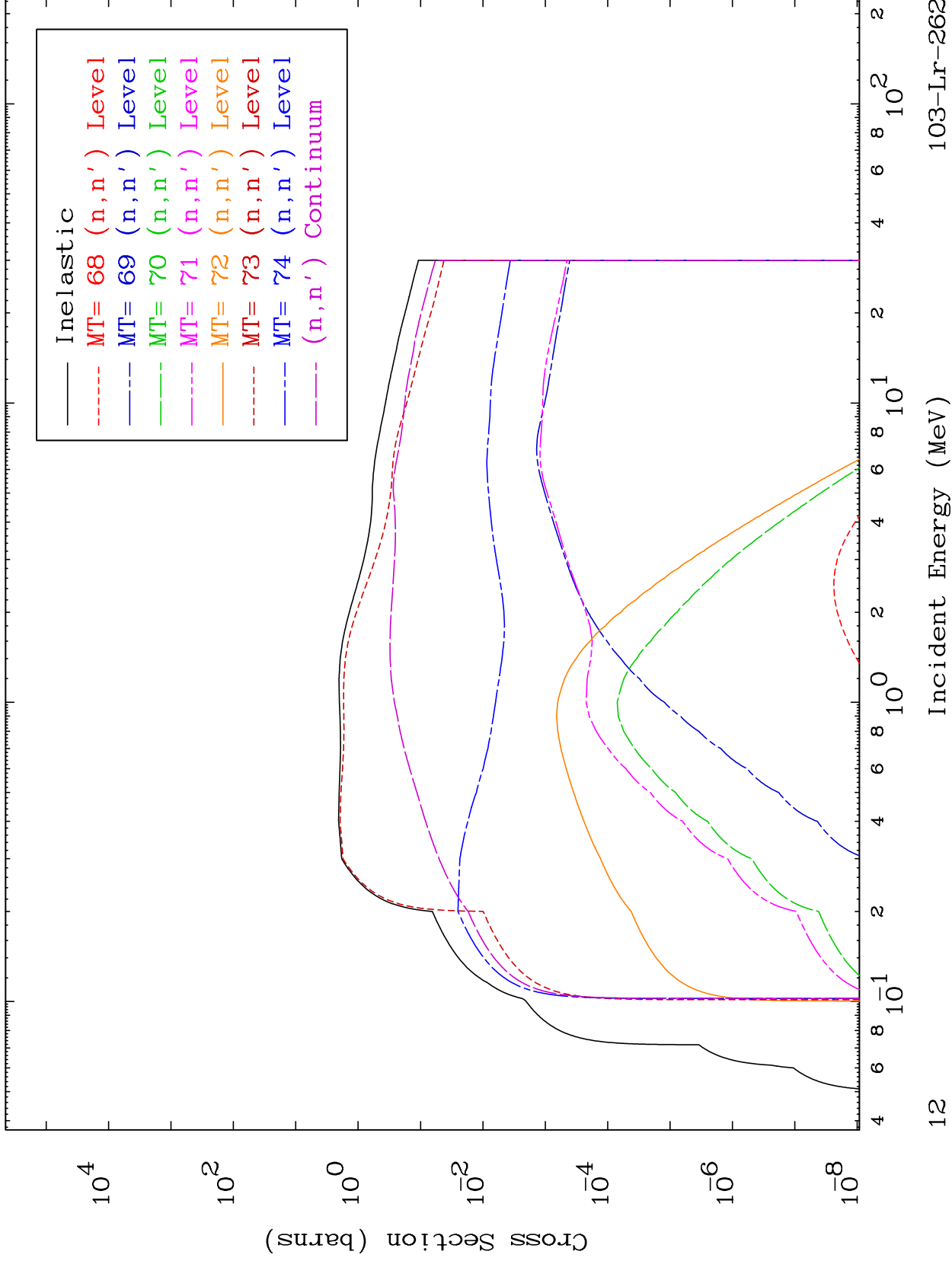


MAT 9991

(n,n') Levels

293 Kelvin Cross Sections

103-Lr-262



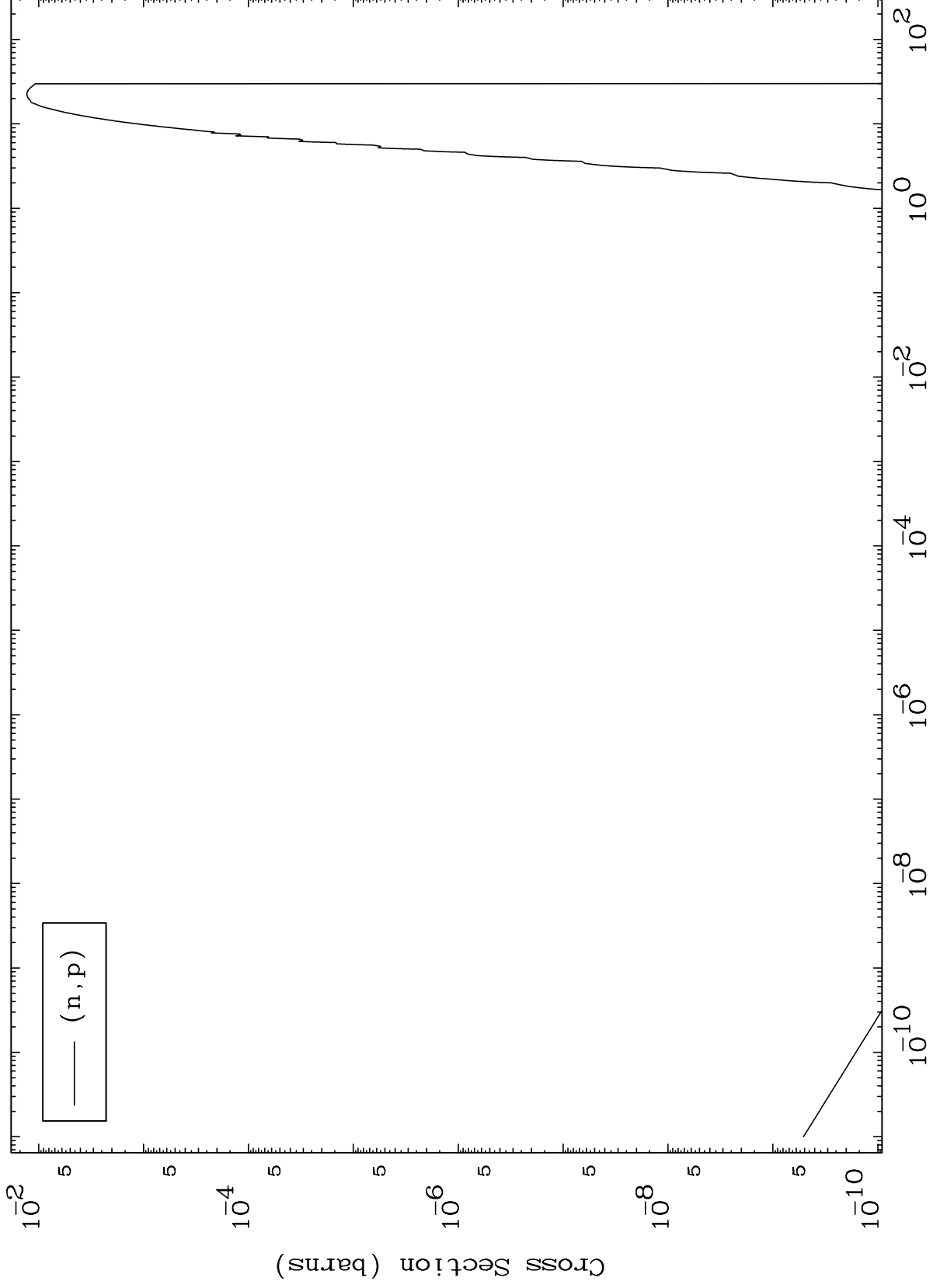
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103-Lr-262

MAT 9991

(n,p) Levels
293 Kelvin Cross Sections

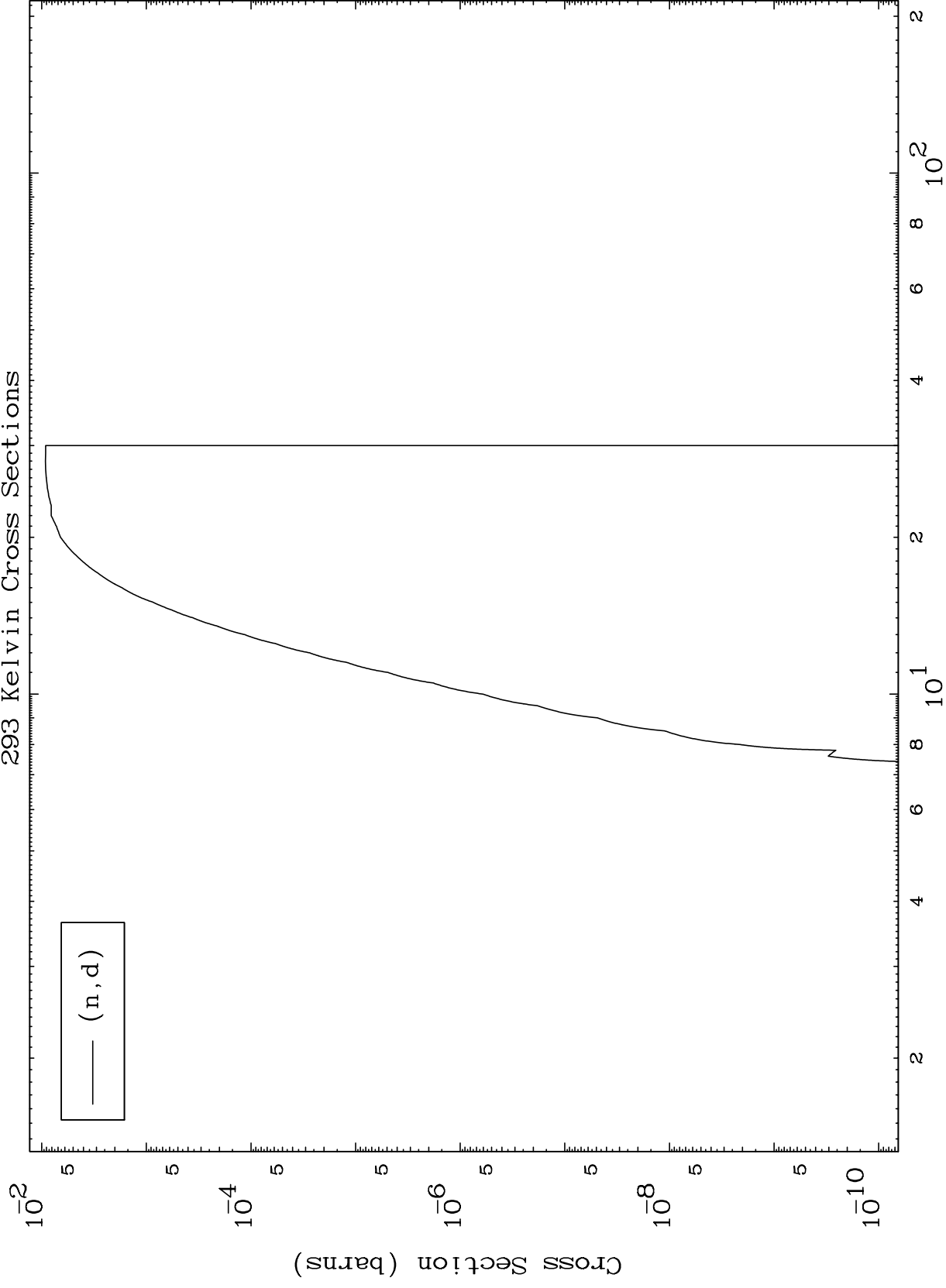
103-Lr-262



MAT 9991

(n,d) Levels
293 Kelvin Cross Sections

103-Lr-262



14

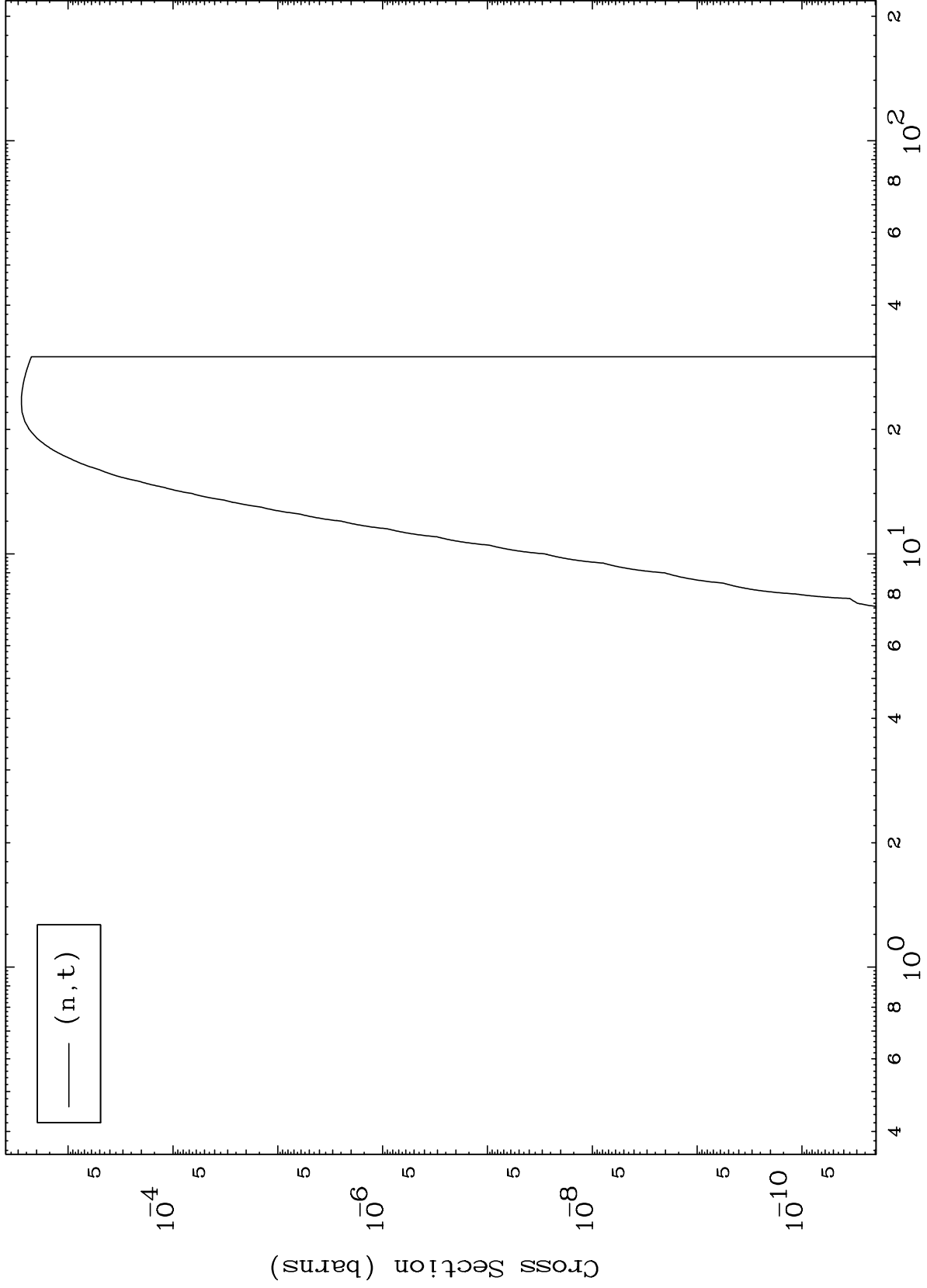
Incident Energy (MeV)

103-Lr-262

MAT 9991

(n,t) Levels
293 Kelvin Cross Sections

103-Lr-262



15

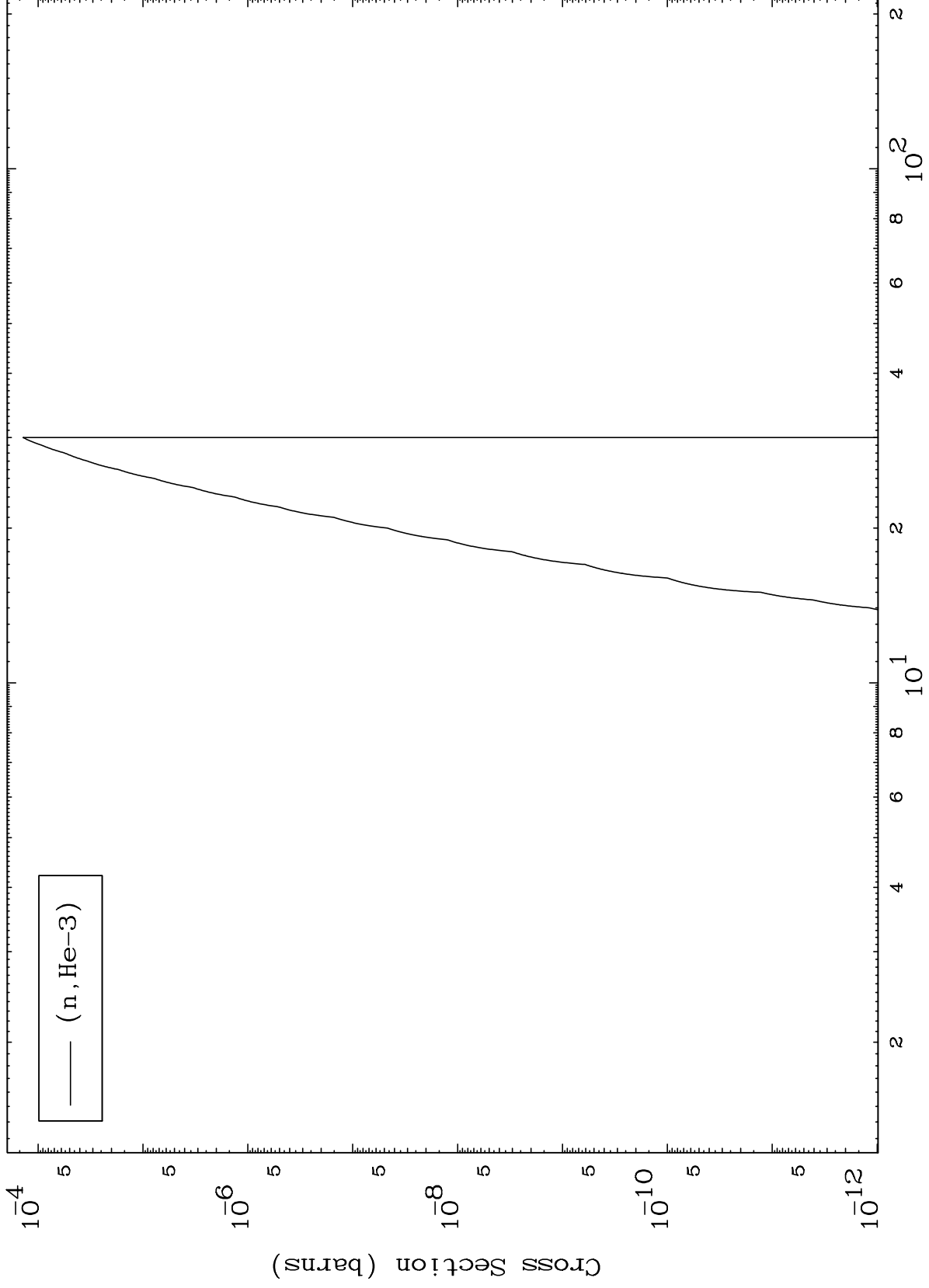
Incident Energy (MeV)

103-Lr-262

MAT 9991

(n,He3) Levels
293 Kelvin Cross Sections

103-Lr-262



16

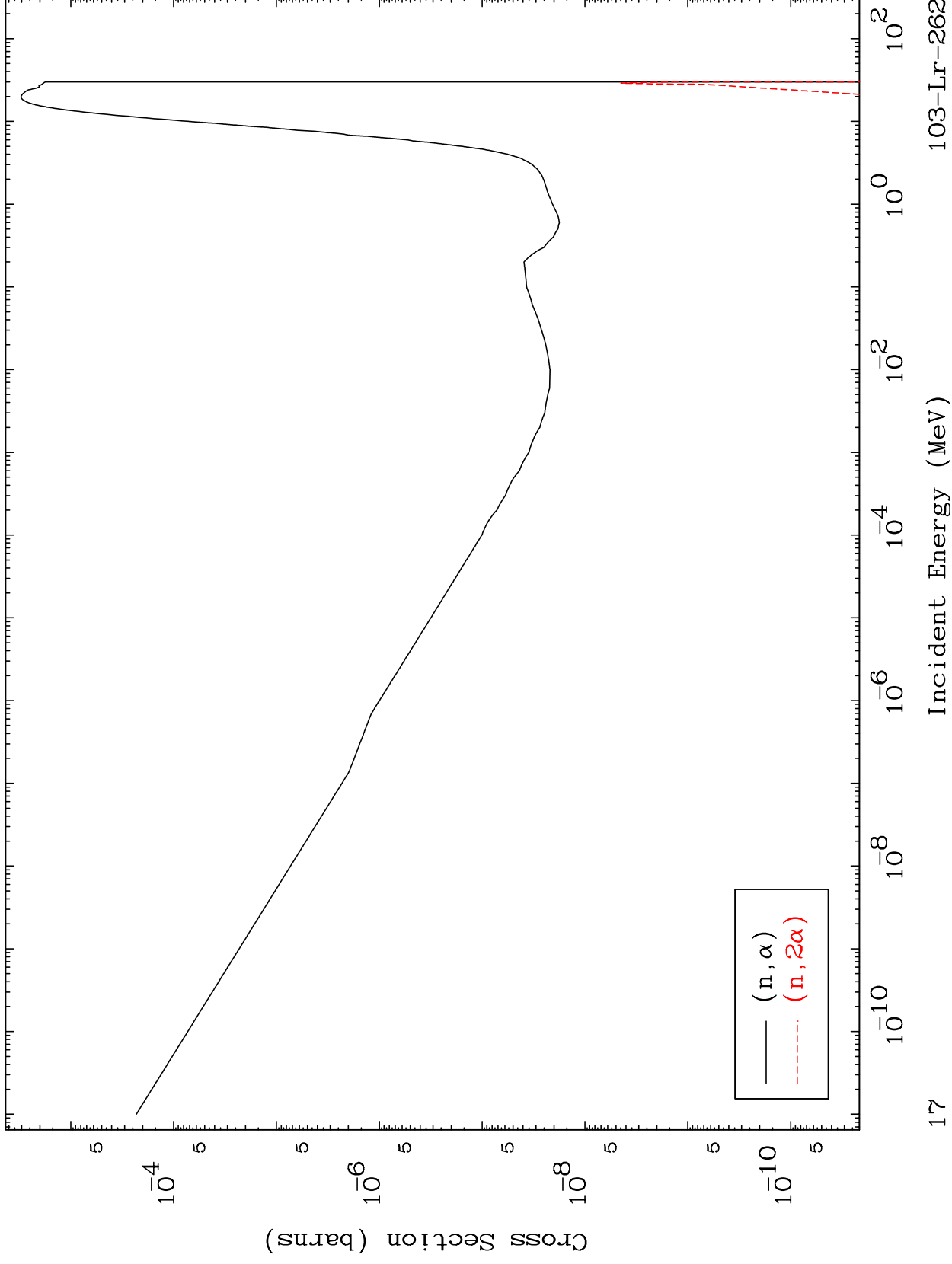
Incident Energy (MeV)

103-Lr-262

MAT 9991

(n, α) Levels
293 Kelvin Cross Sections

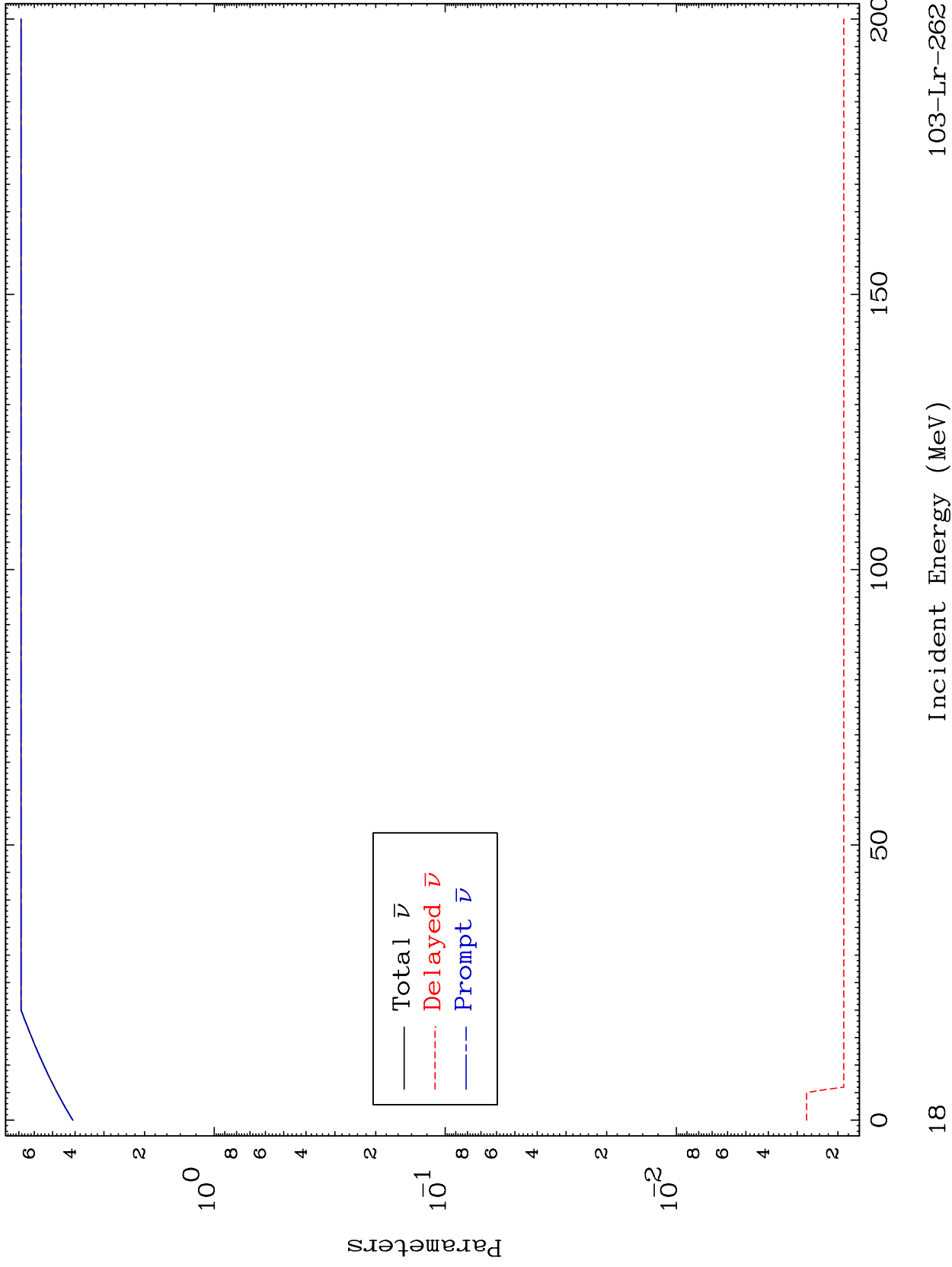
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Energy Release
Parameters

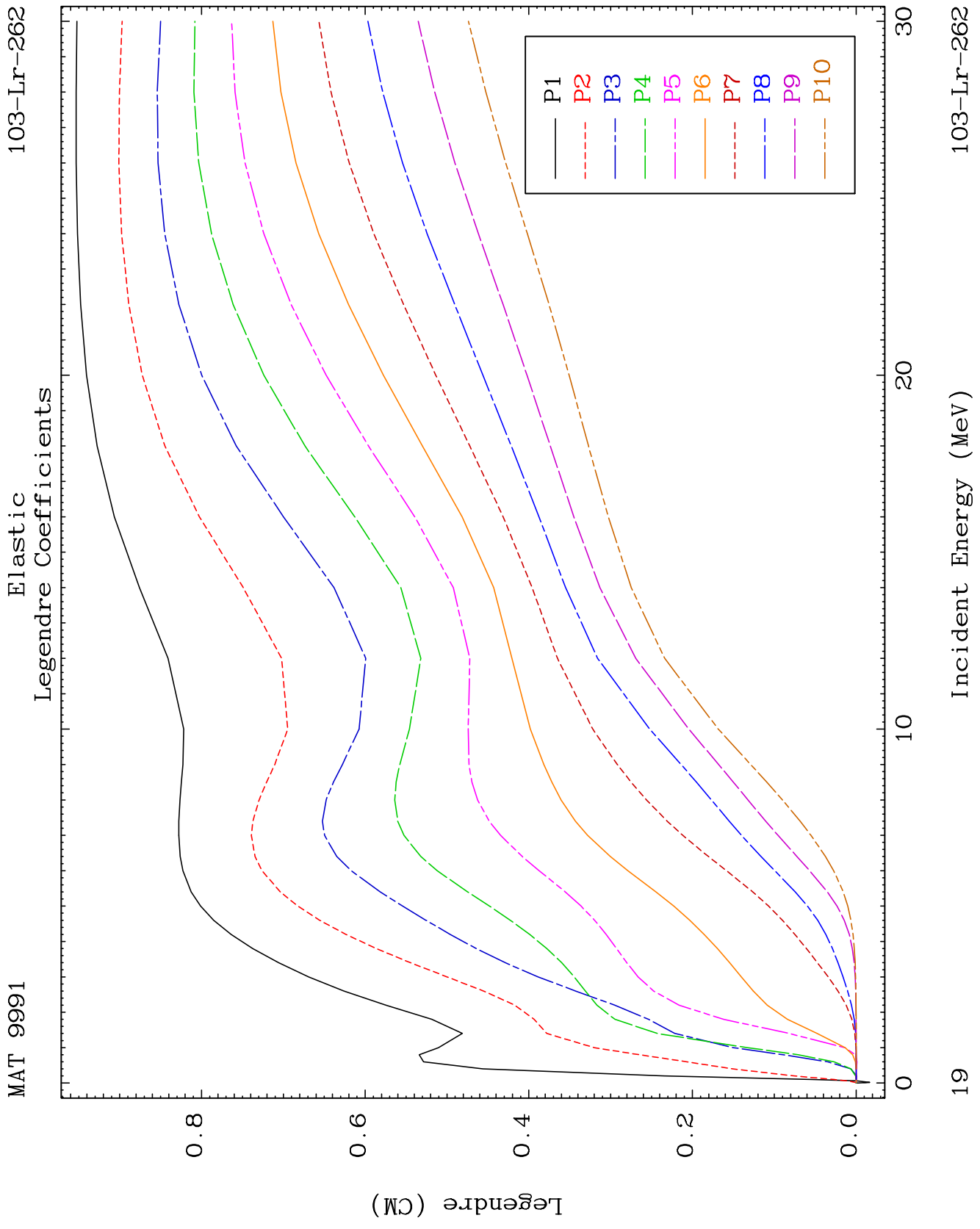
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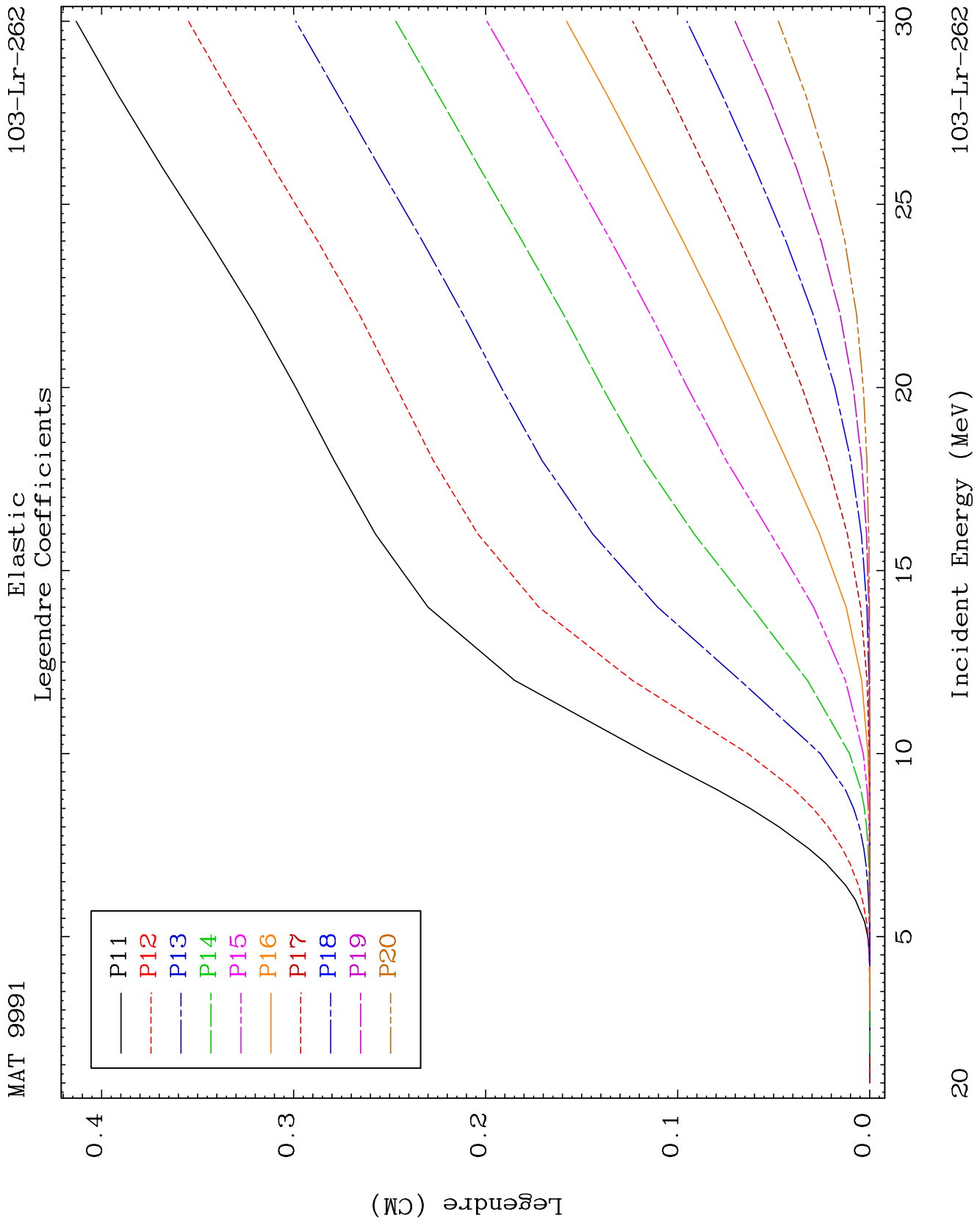


18

Incident Energy (MeV)

103-Lr-262

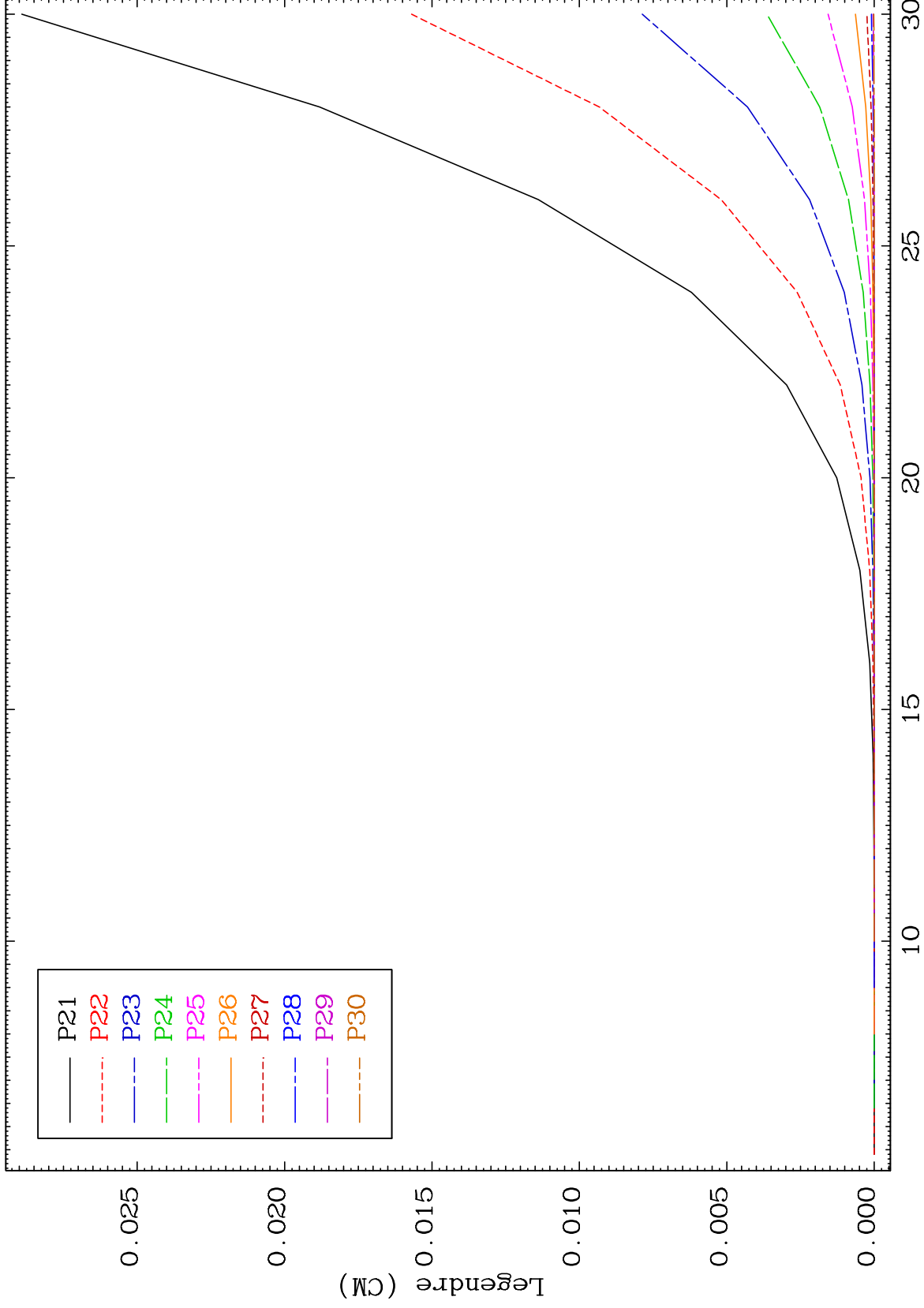




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Elastic Legendre Coefficients

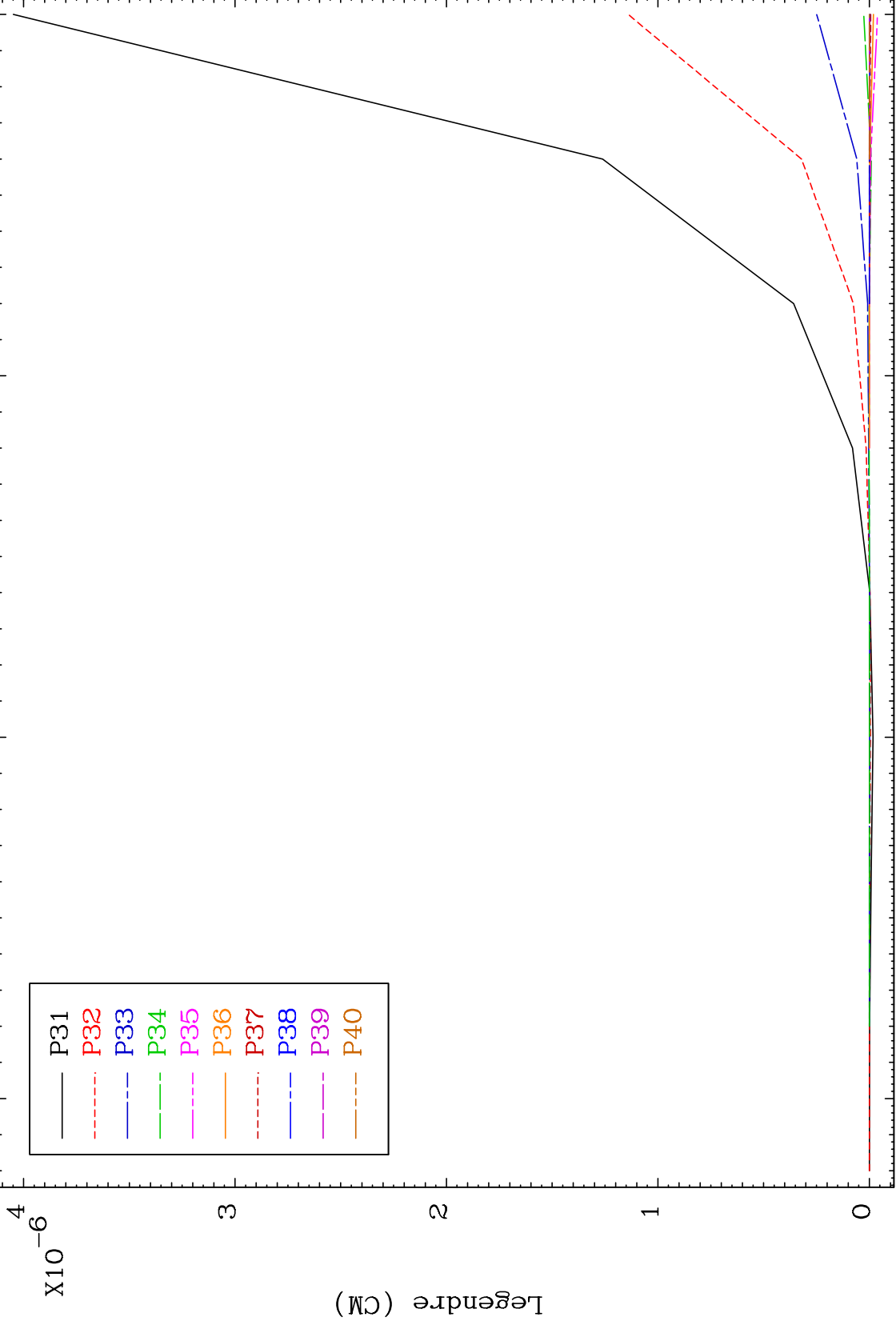
103-Lr-262



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Elastic Legendre Coefficients

103-Lr-262



22

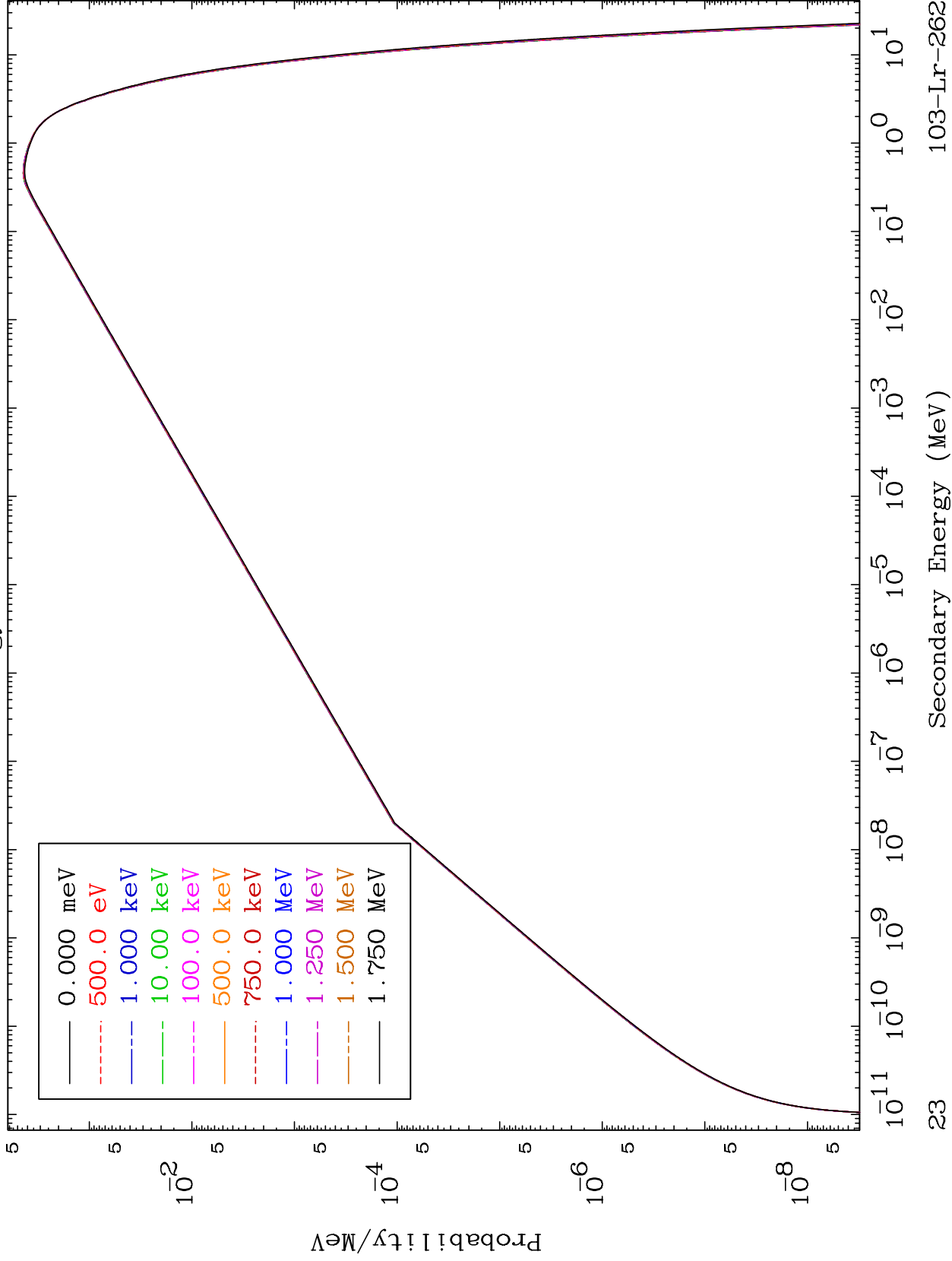
Incident Energy (MeV)

103-Lr-262

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Fission Energy Distributions

103-Lr-262



103-Lr-262

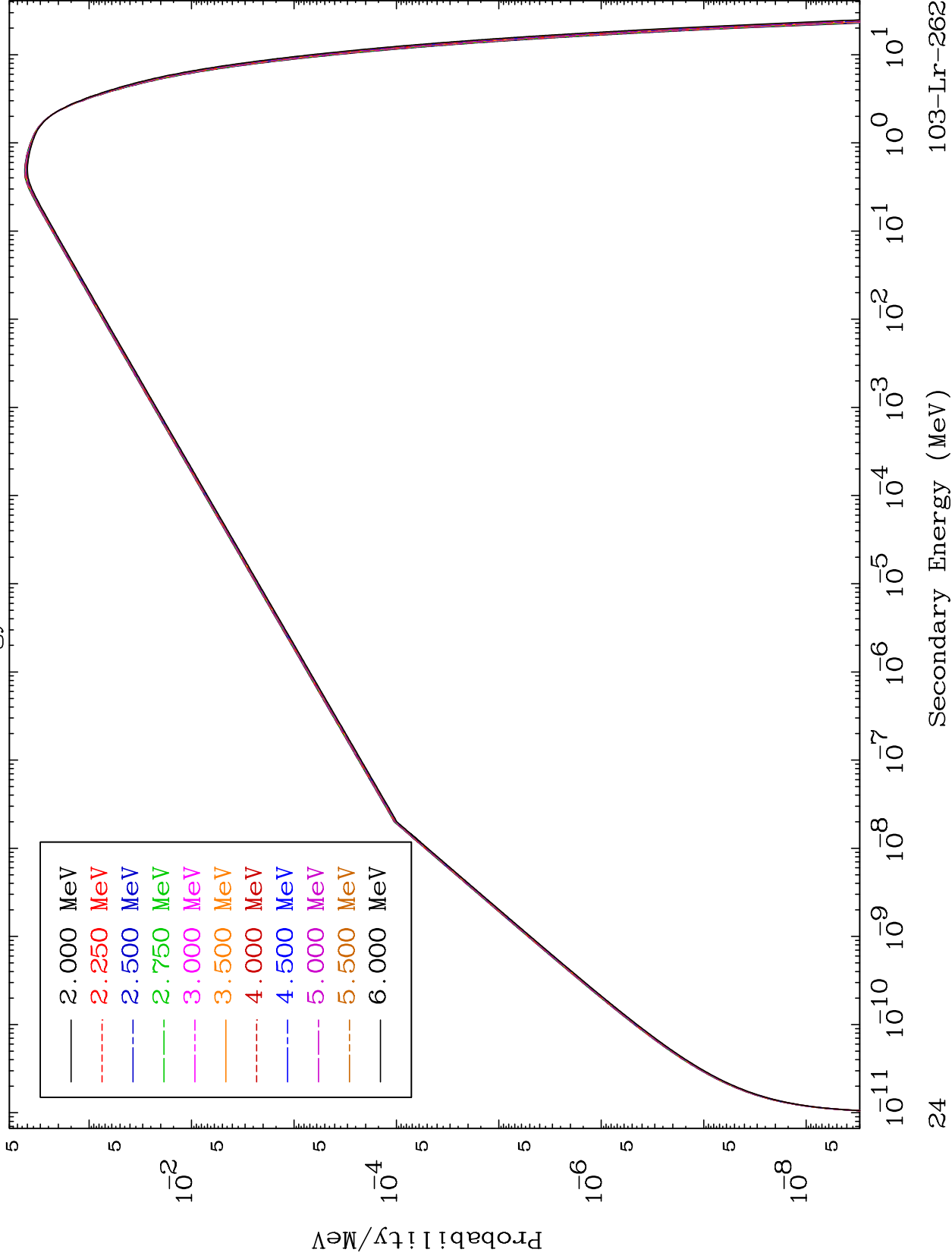
Secondary Energy (MeV)

23

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Fission Energy Distributions

103-Lr-262



103-Lr-262

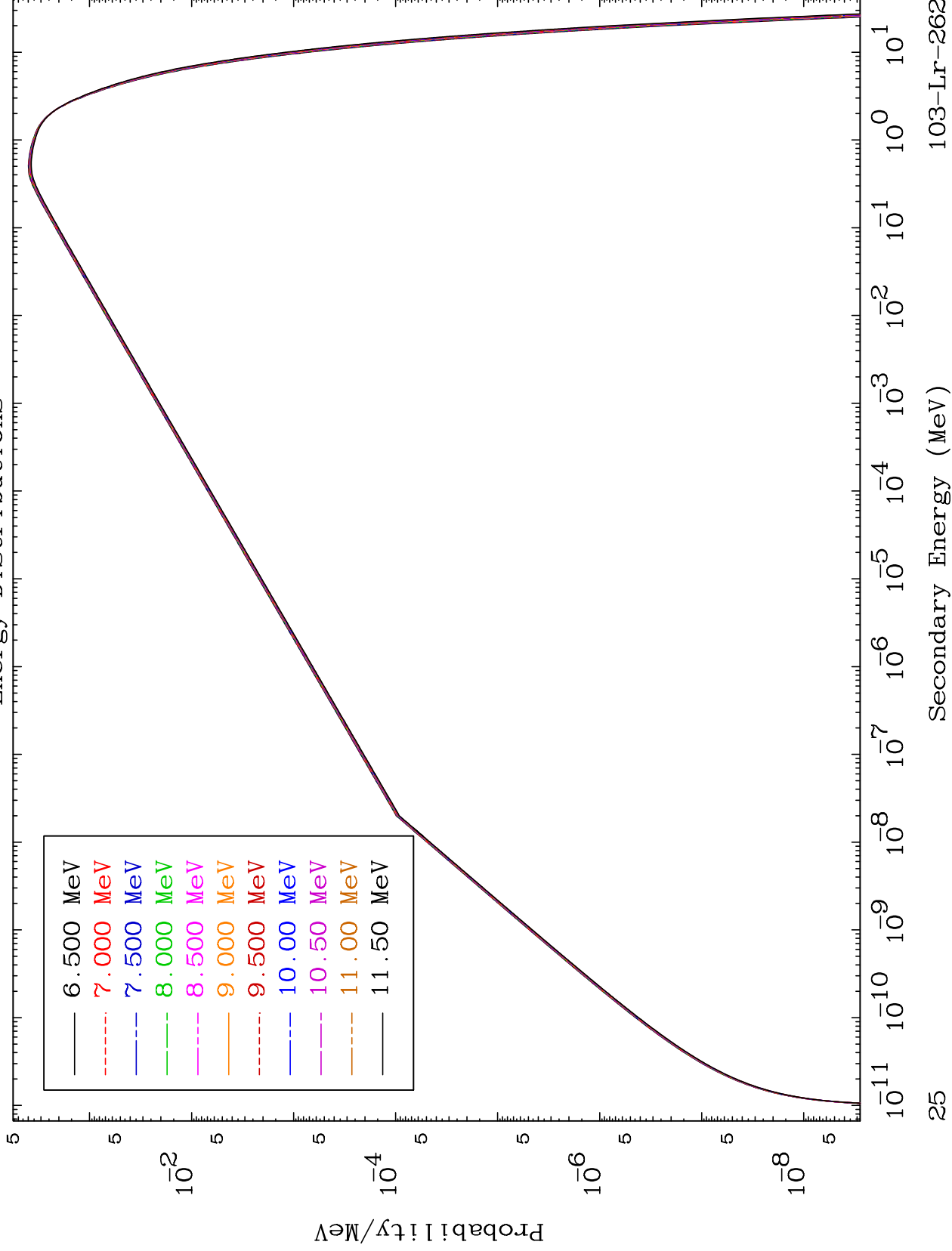
Secondary Energy (MeV)

24

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Fission
Energy Distributions

103-Lr-262



25

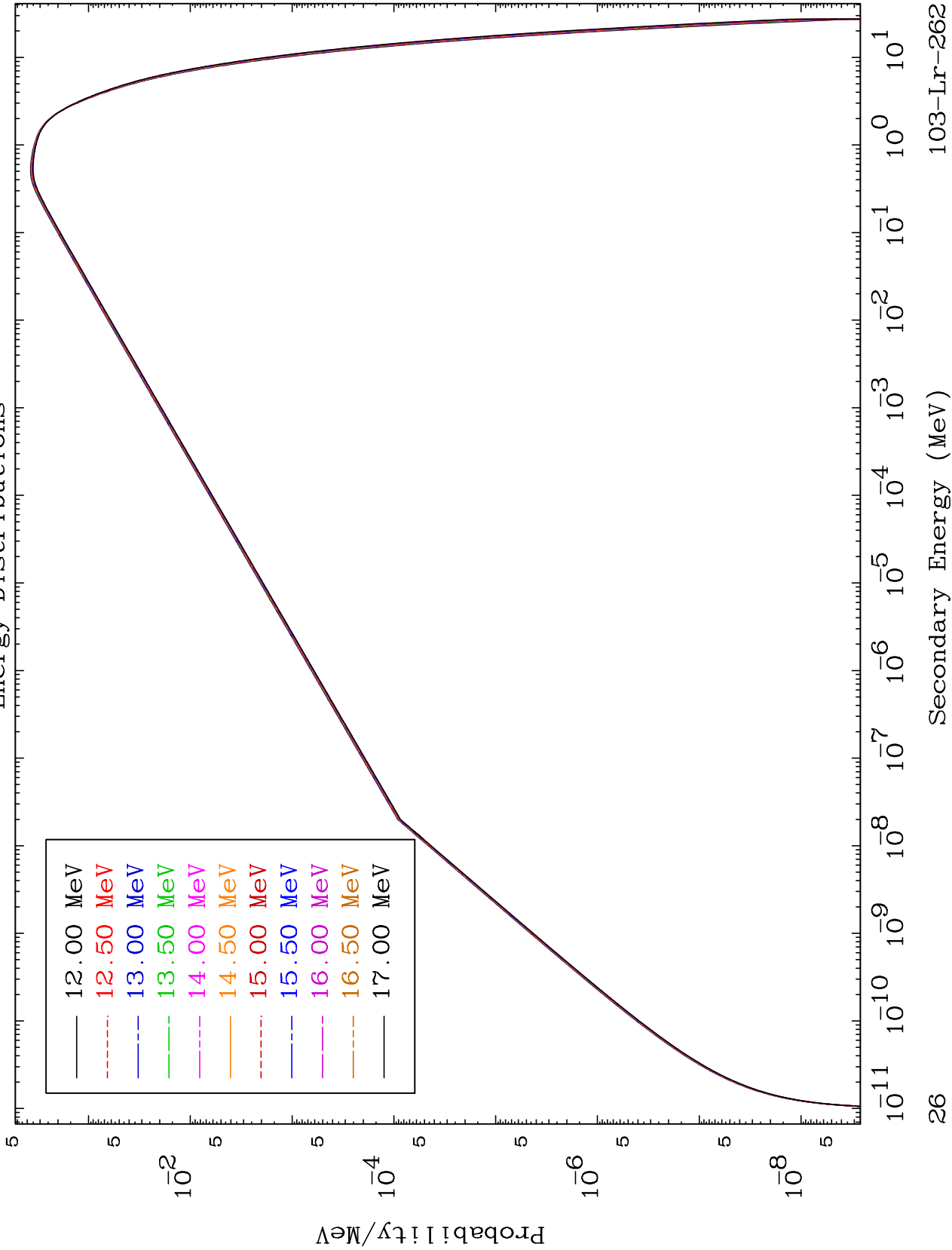
Secondary Energy (MeV)

103-Lr-262

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Fission
Energy Distributions

103-Lr-262

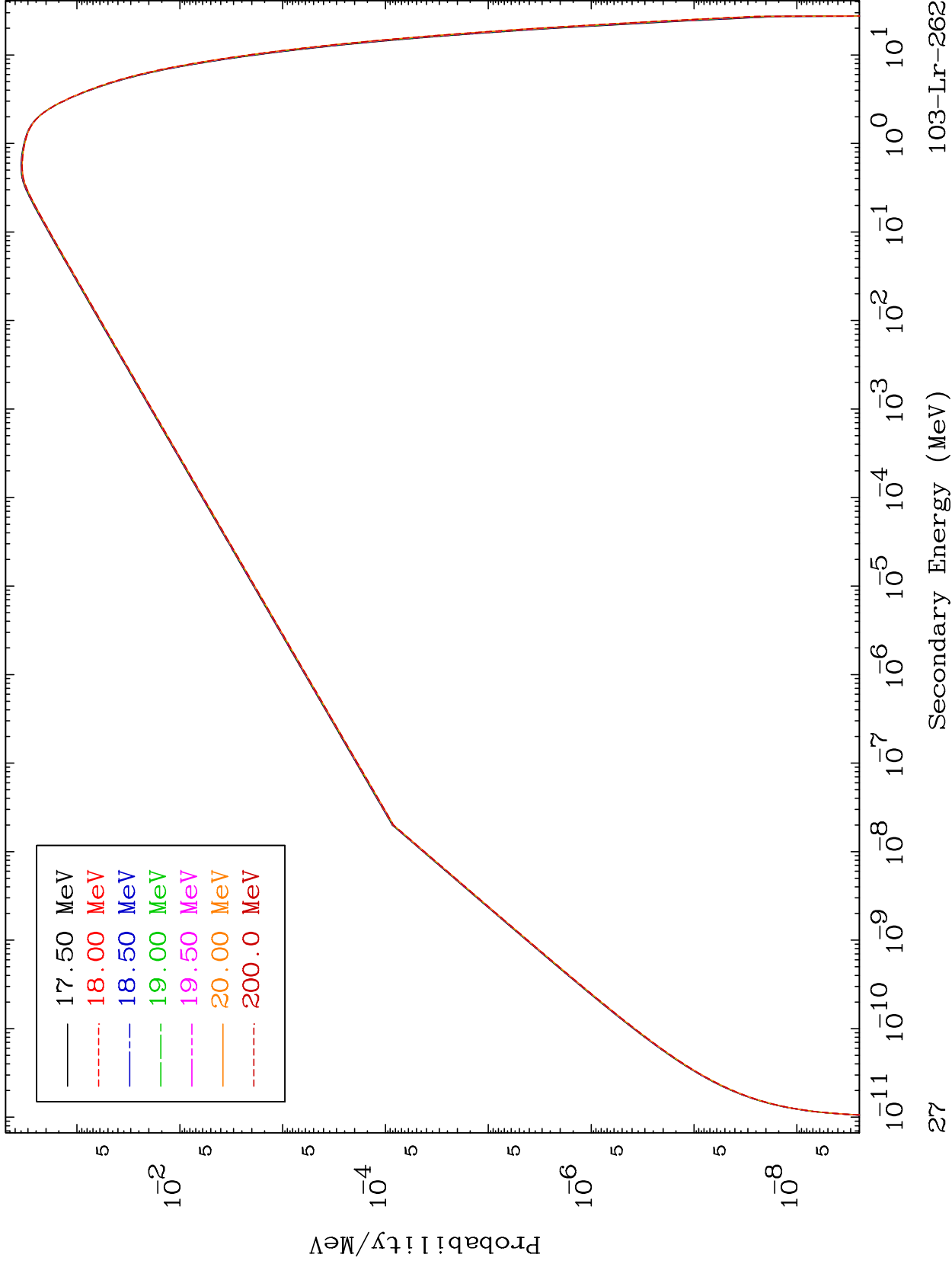


103-Lr-262

Secondary Energy (MeV)

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Fission Energy Distributions



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$(n, n') \alpha$

103-Lr-262

Radionuclide Production Cross Section

