

Program EVALPLOT
(Version 2021-1)

by

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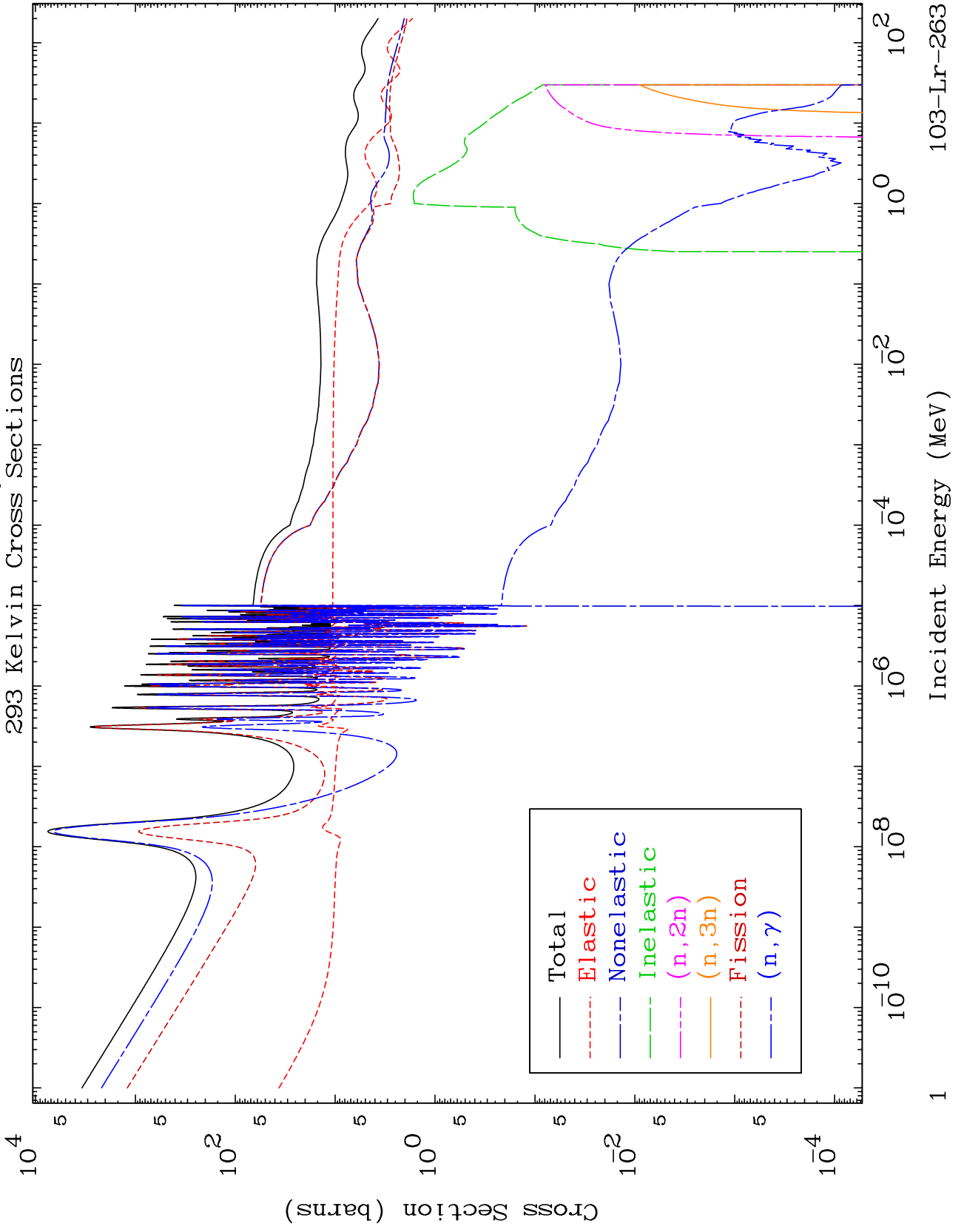
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Press Mouse Button to Start

MAT 9992

Neutron Major
293 Kelvin Cross Sections

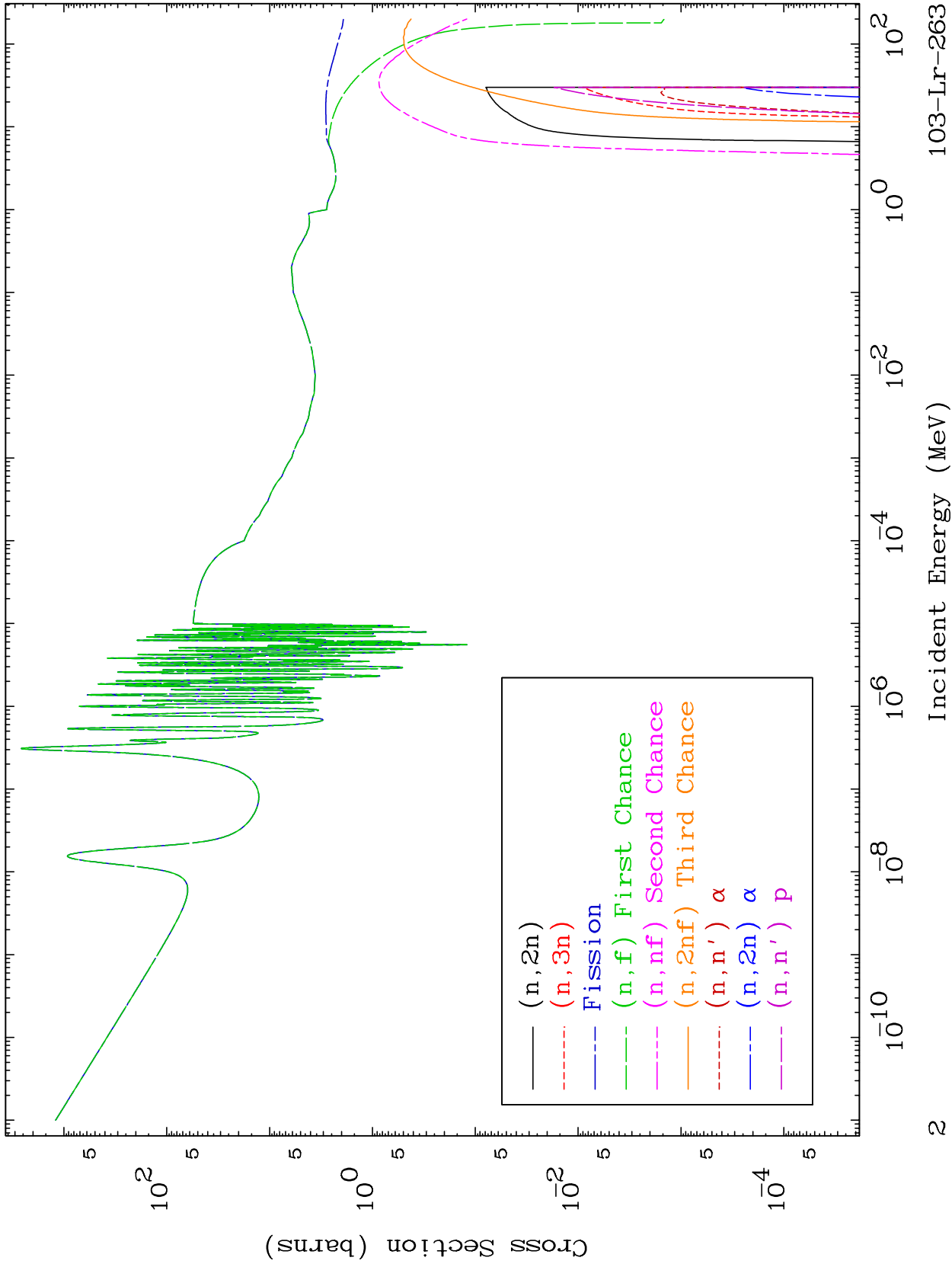
103-Lr-263

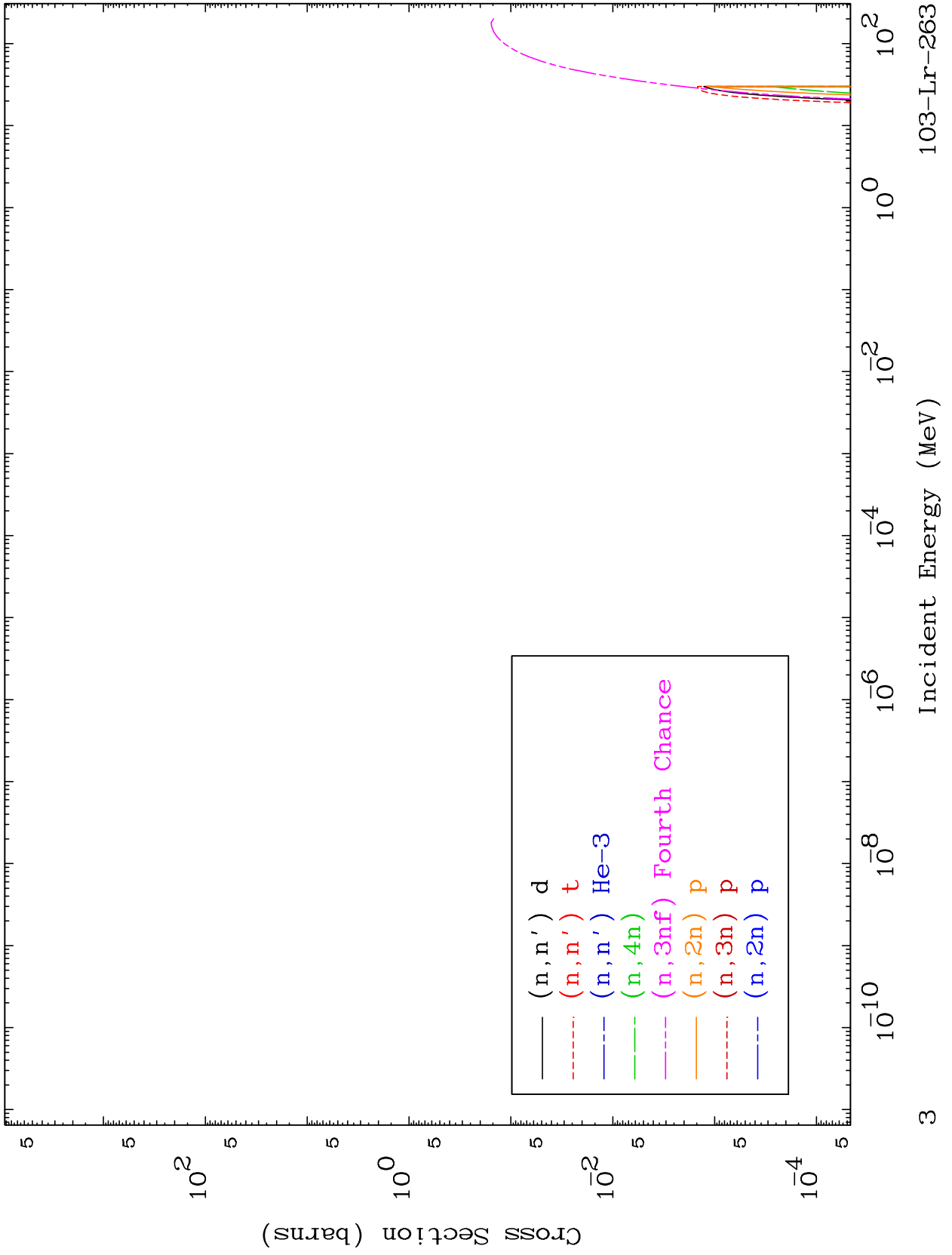


MAT 9992

Neutron Absorption
293 Kelvin Cross Sections

103-Lr-263

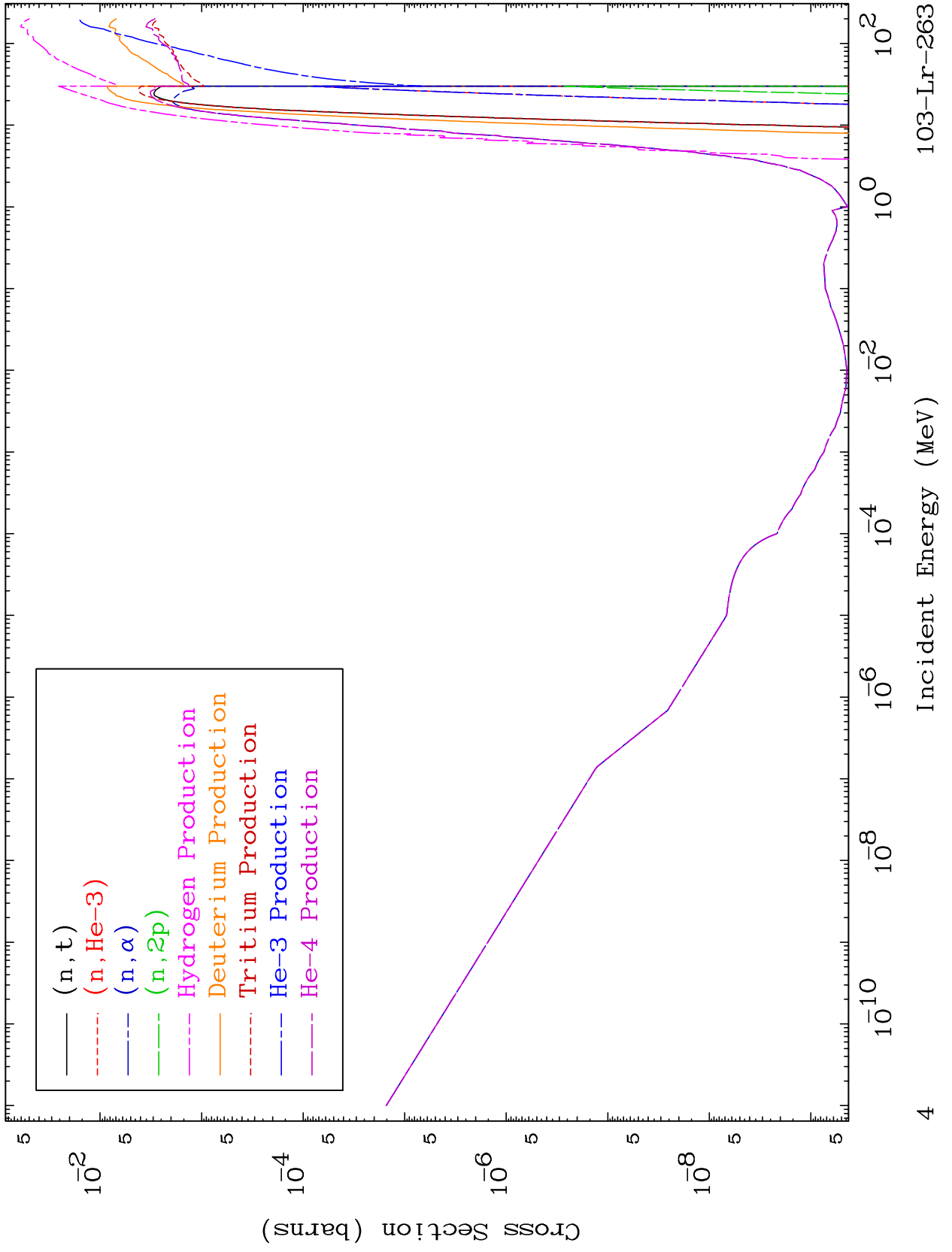




MAT 9992

Neutron Absorption
293 Kelvin Cross Sections

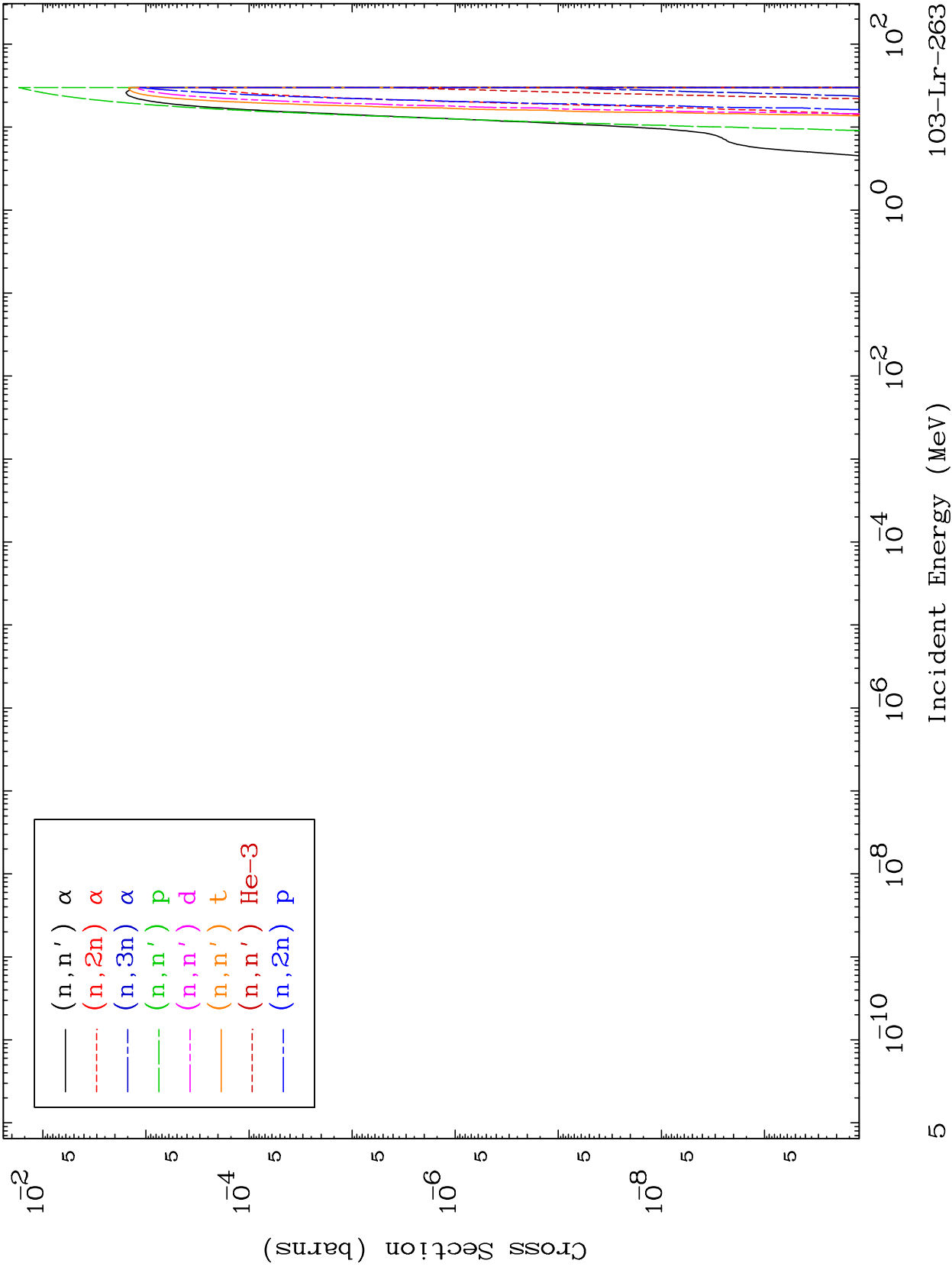
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Charged Particle
293 Kelvin Cross Sections

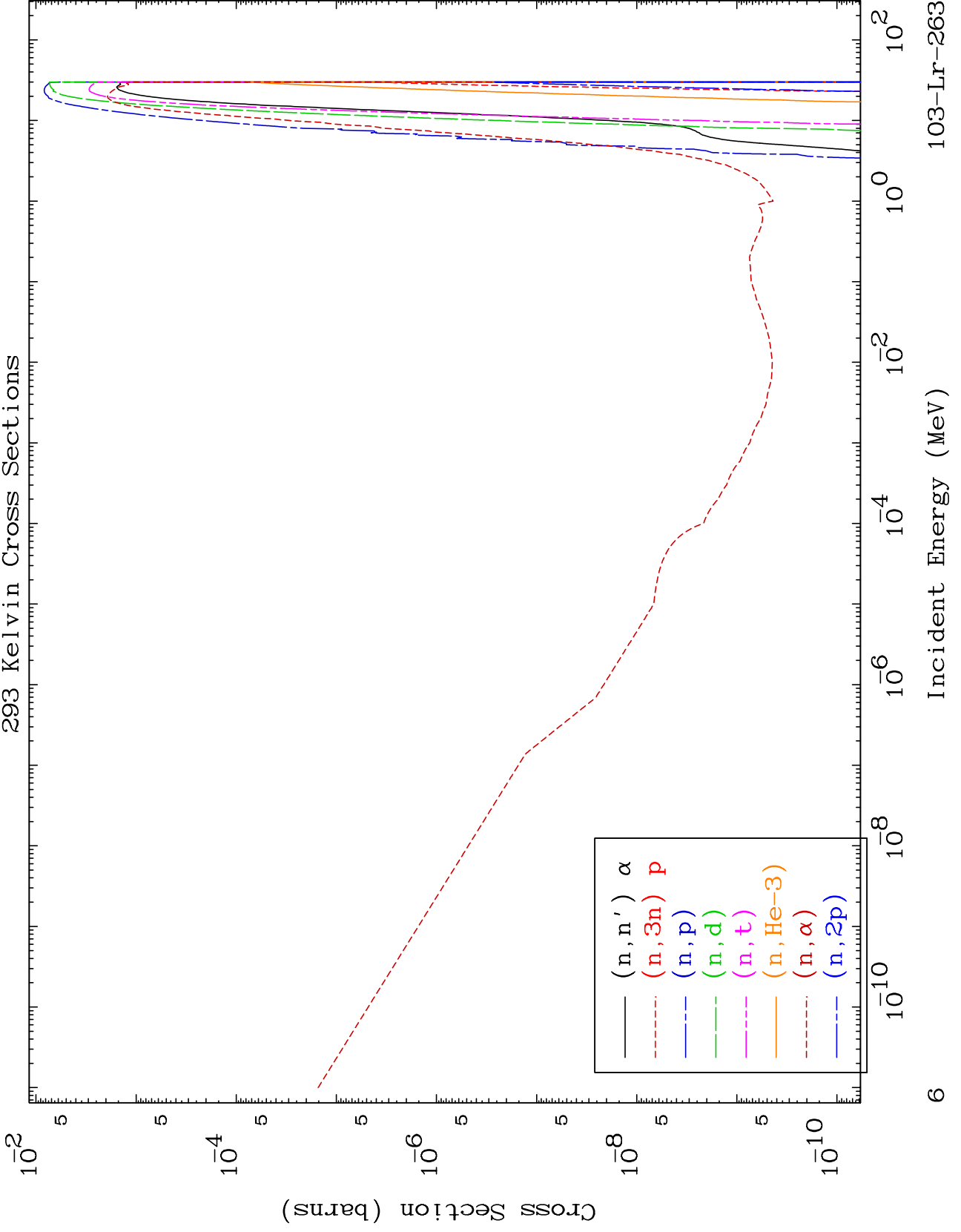
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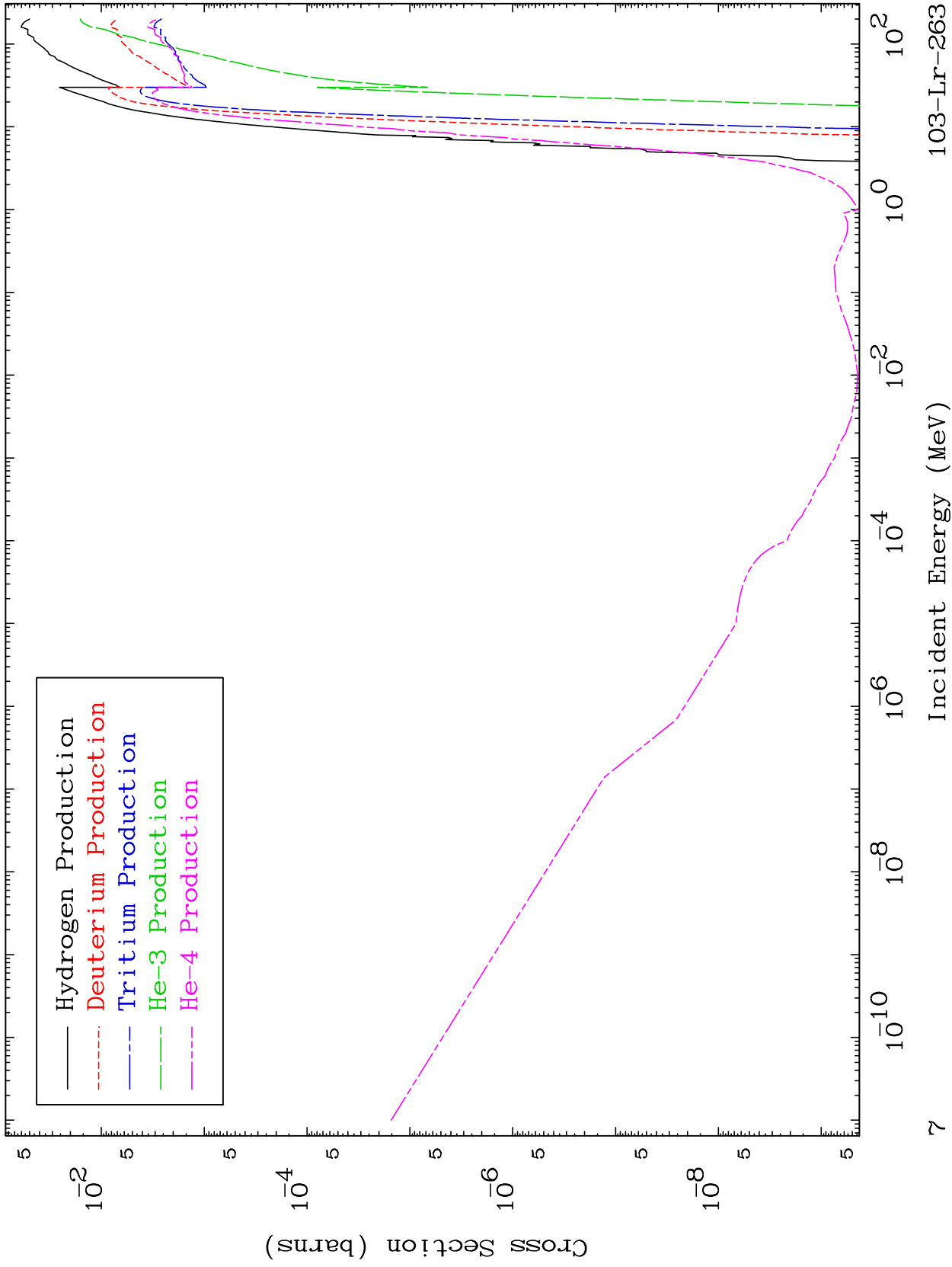


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Charged Particle
293 Kelvin Cross Sections

103-Lr-263

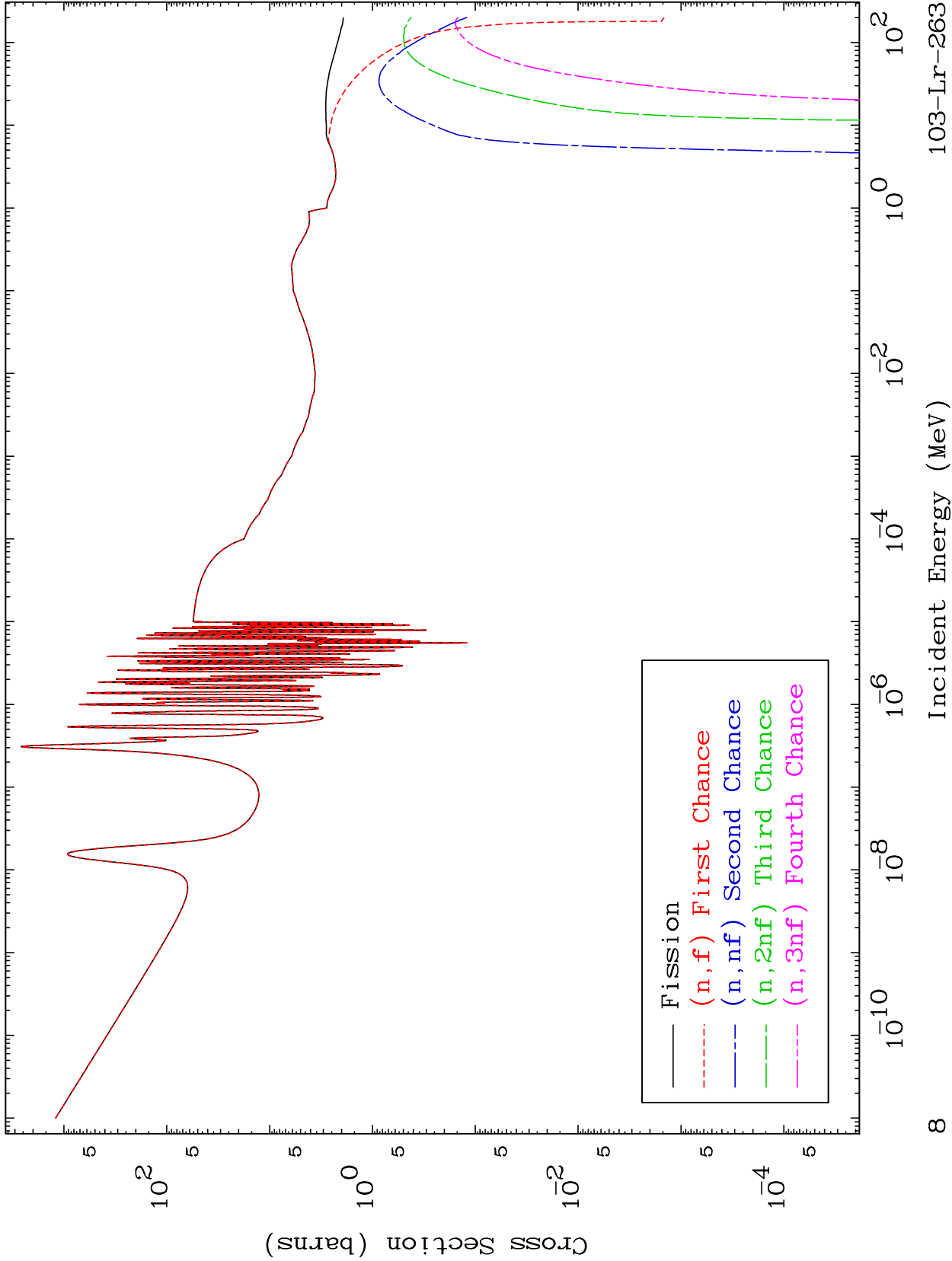


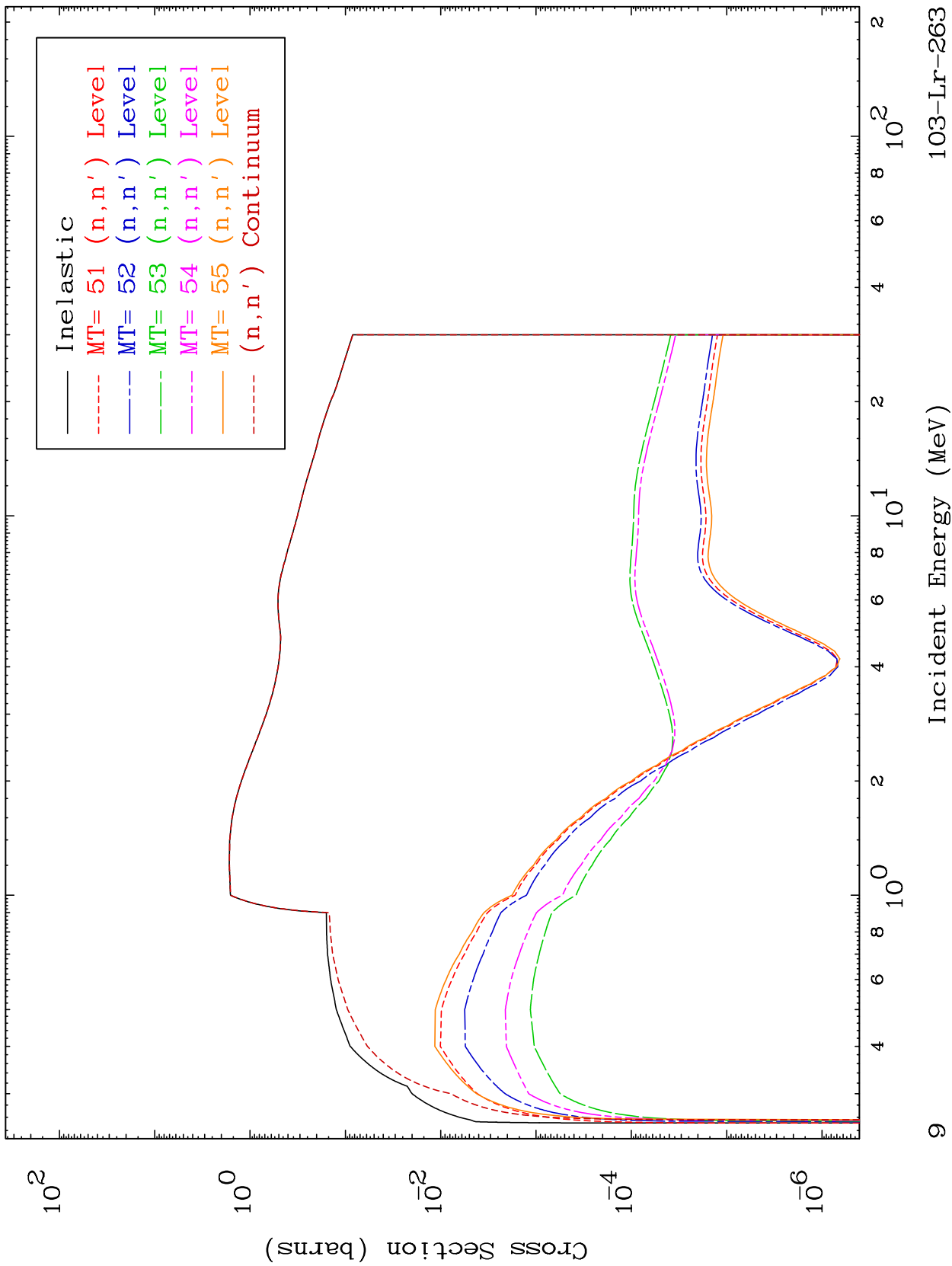


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Fission
293 Kelvin Cross Sections

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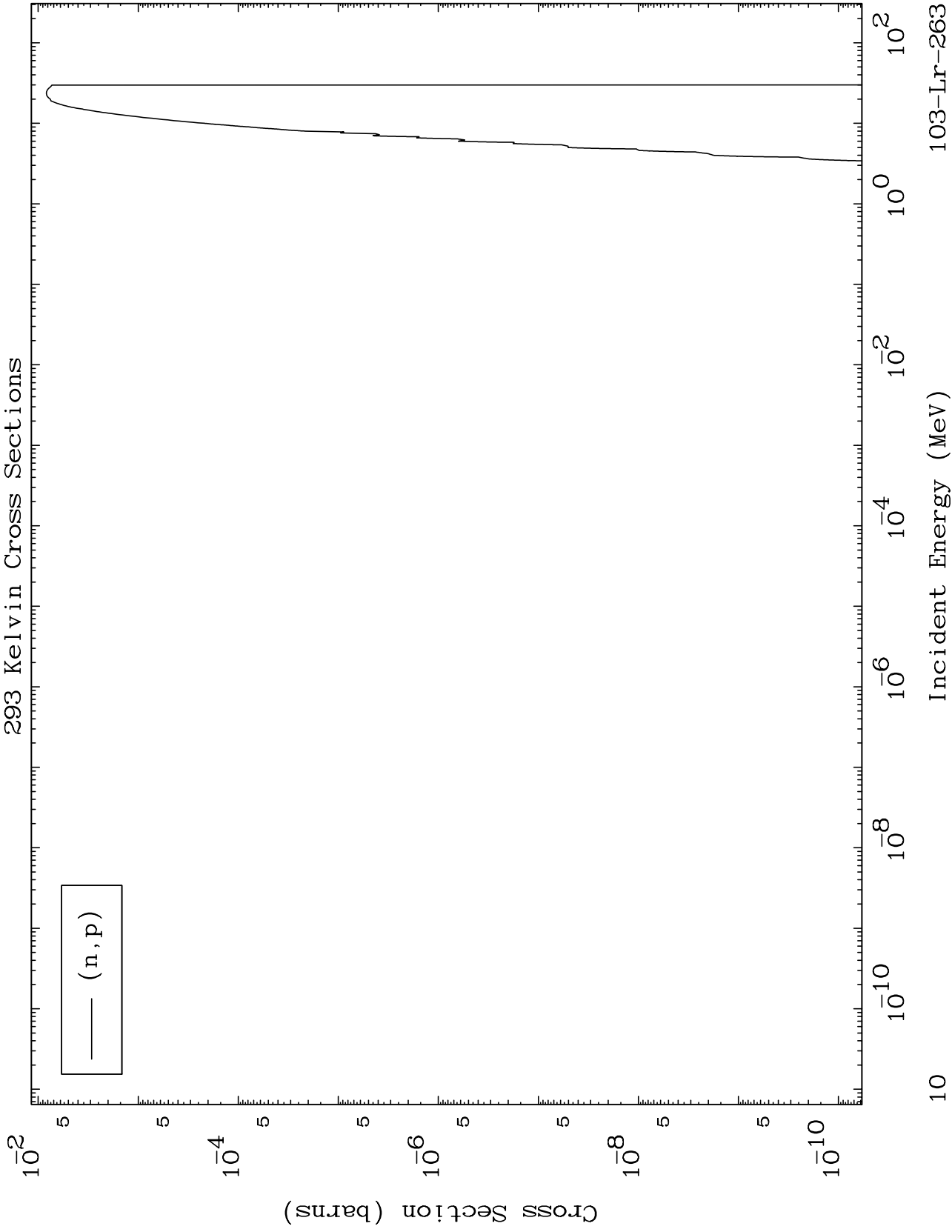




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(n,p) Levels
293 Kelvin Cross Sections

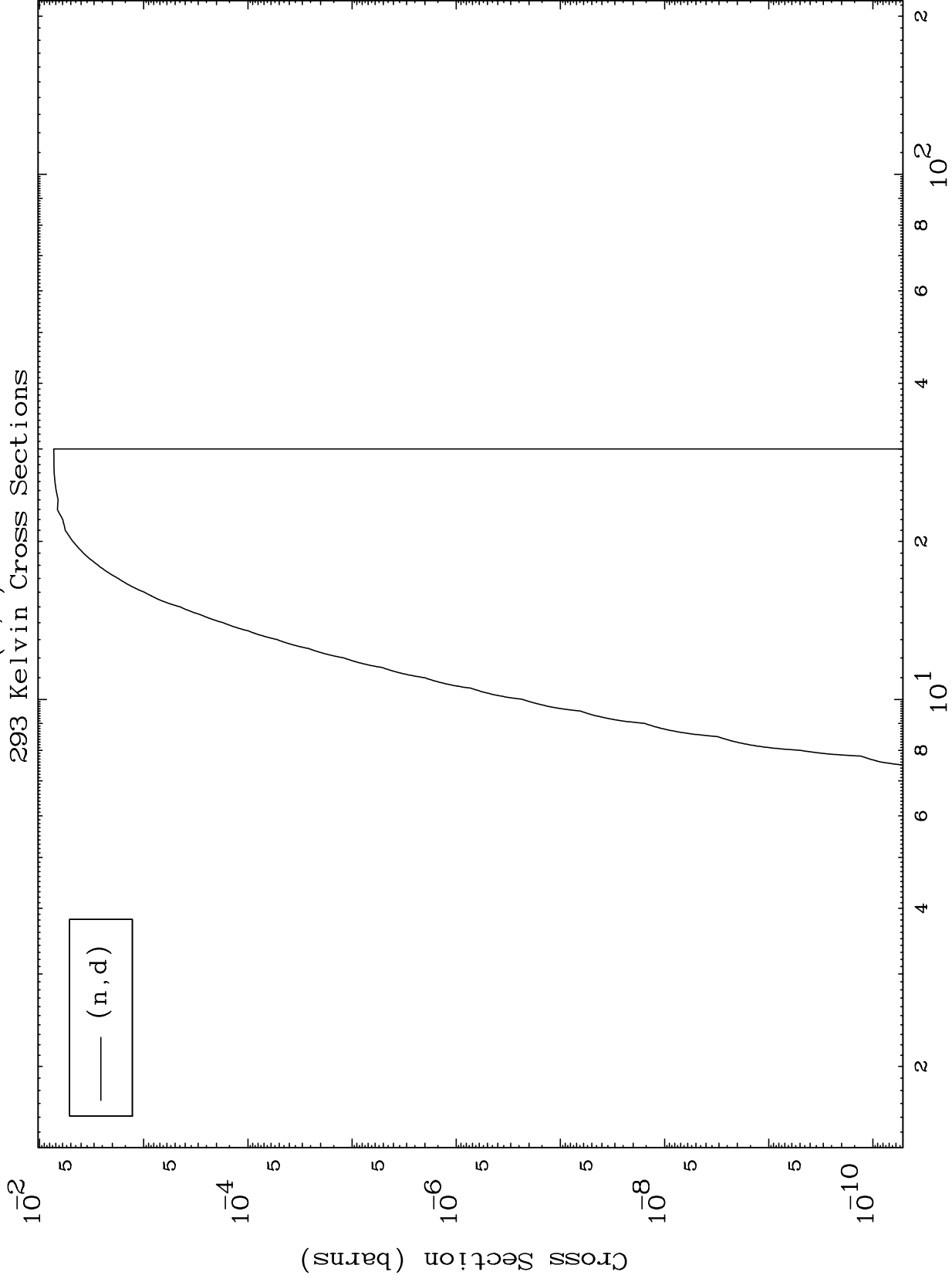
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(n,d) Levels
293 Kelvin Cross Sections

103-Lr-263



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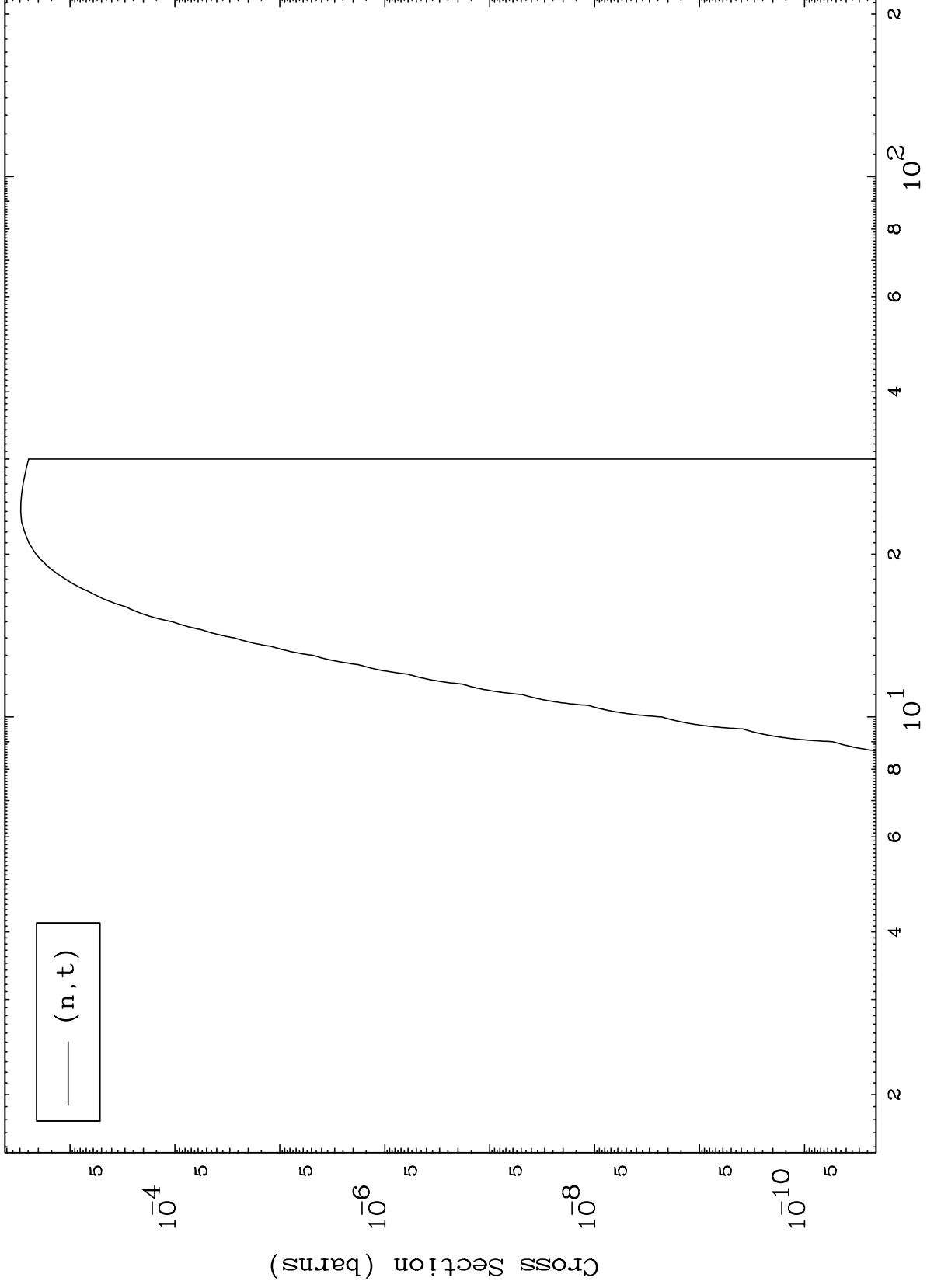
Incident Energy (MeV)

103-Lr-263

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(n,t) Levels
293 Kelvin Cross Sections

103-Lr-263



12

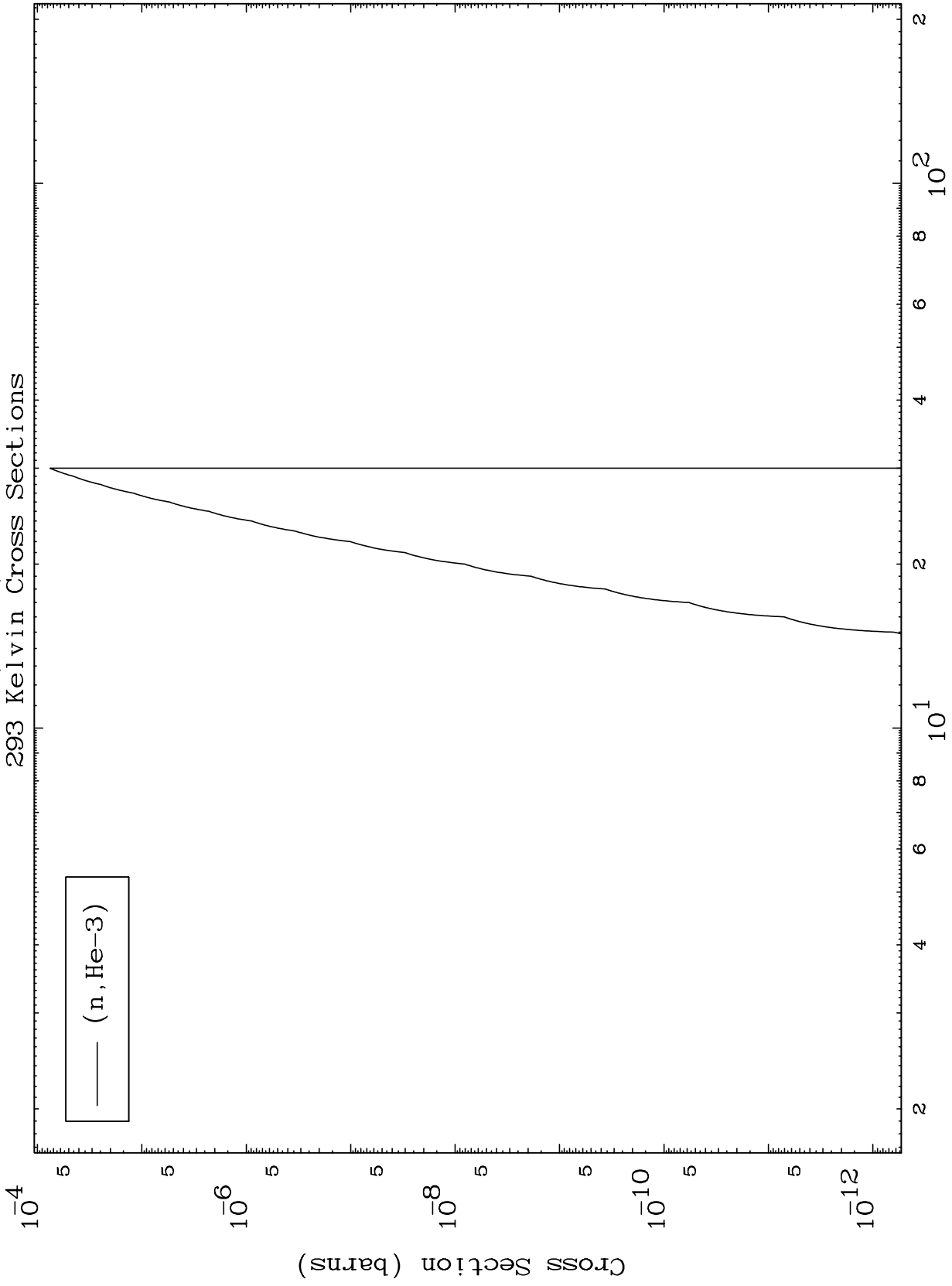
Incident Energy (MeV)

103-Lr-263

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(n,He3) Levels
293 Kelvin Cross Sections

103-Lr-263



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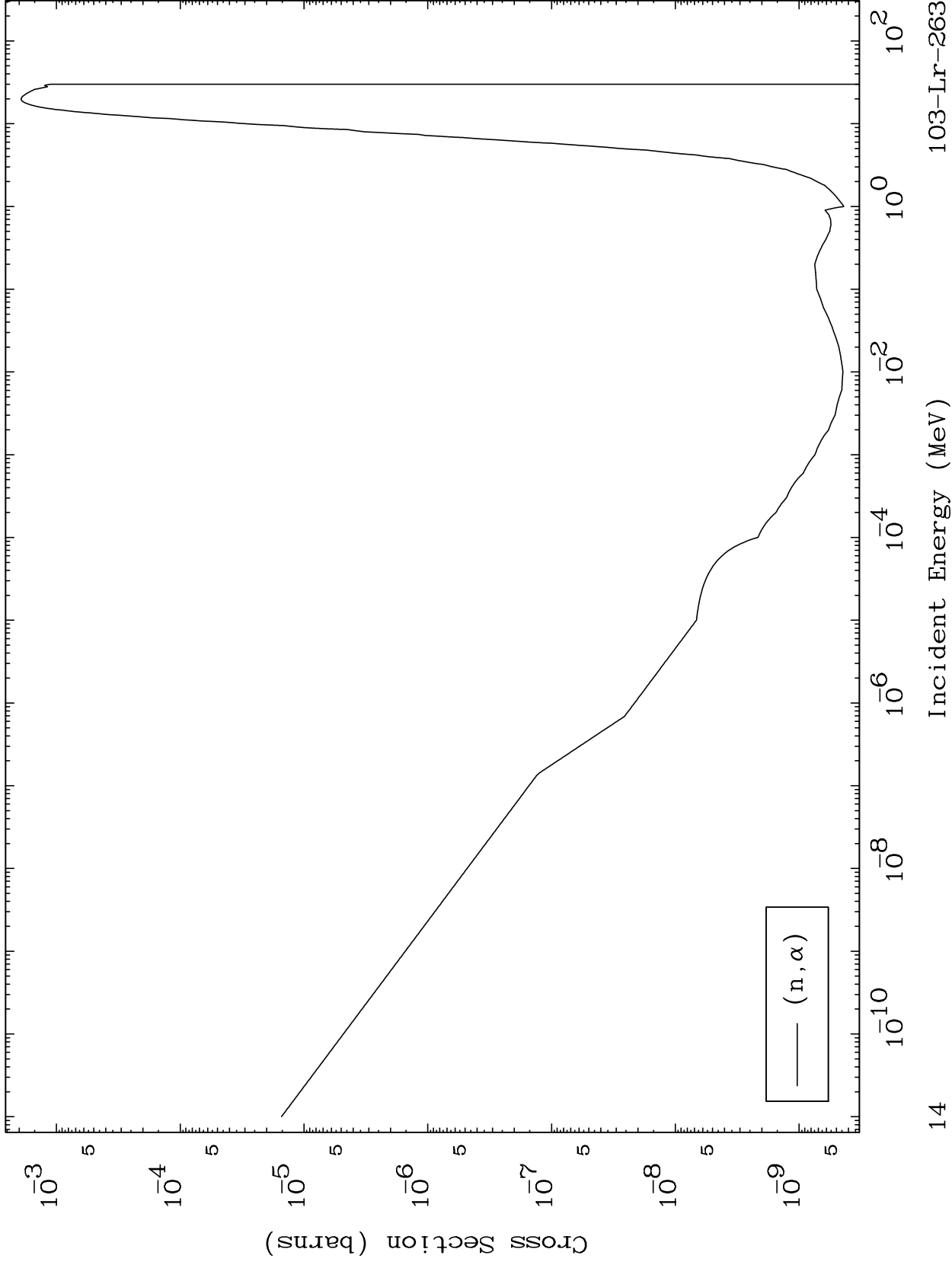
Incident Energy (MeV)

103-Lr-263

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(n, α) Levels
293 Kelvin Cross Sections

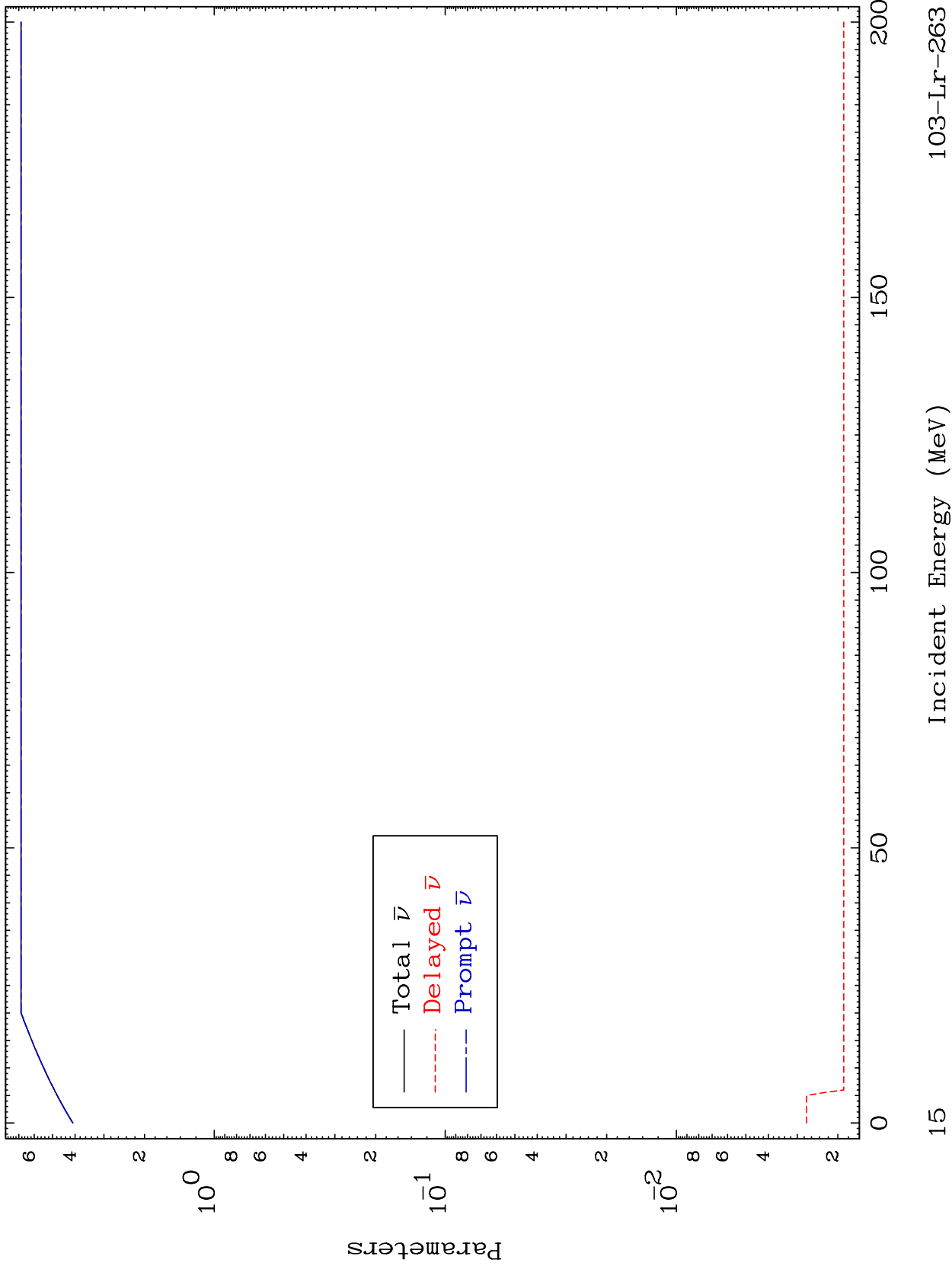
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Energy Release
Parameters

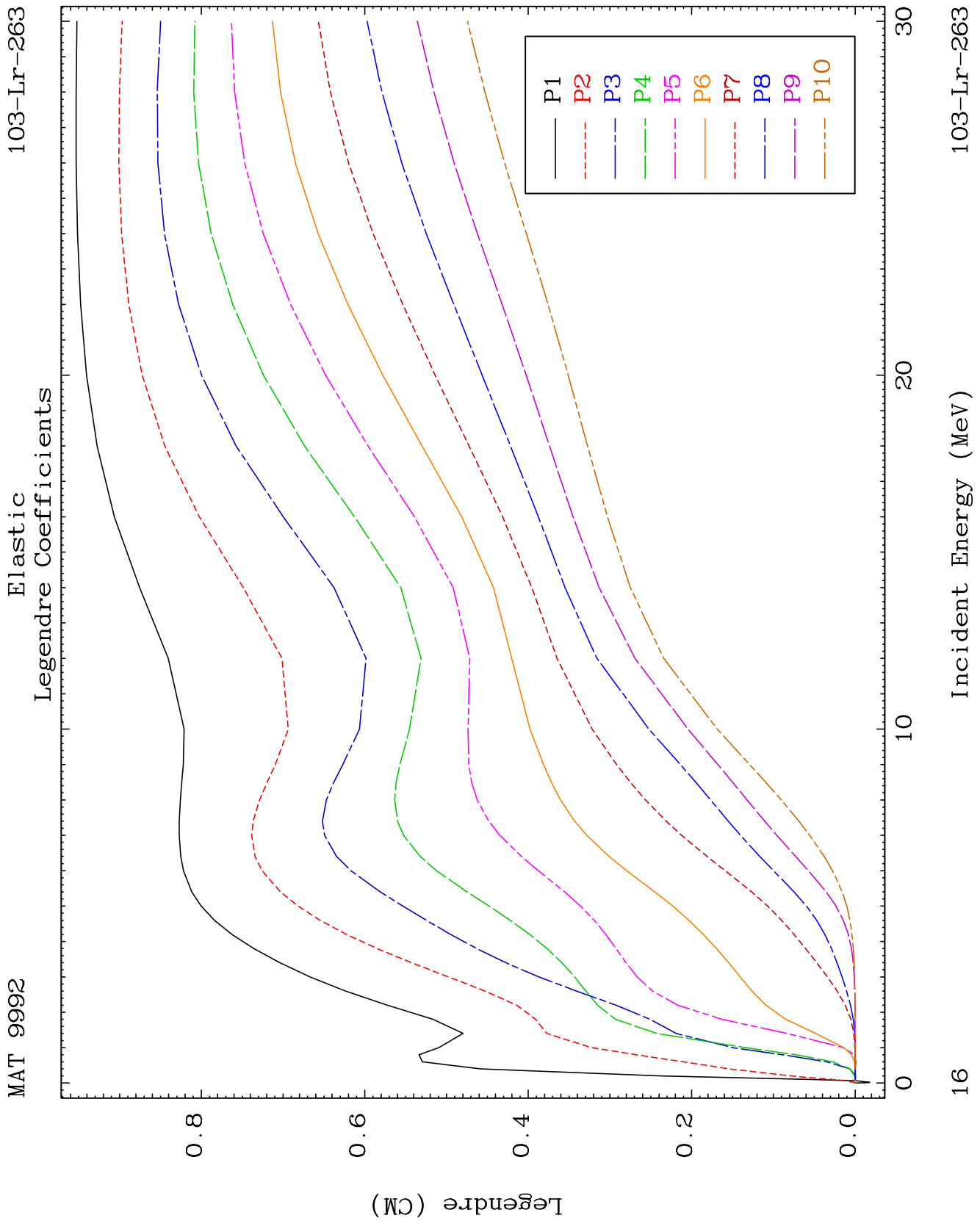
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Incident Energy (MeV)

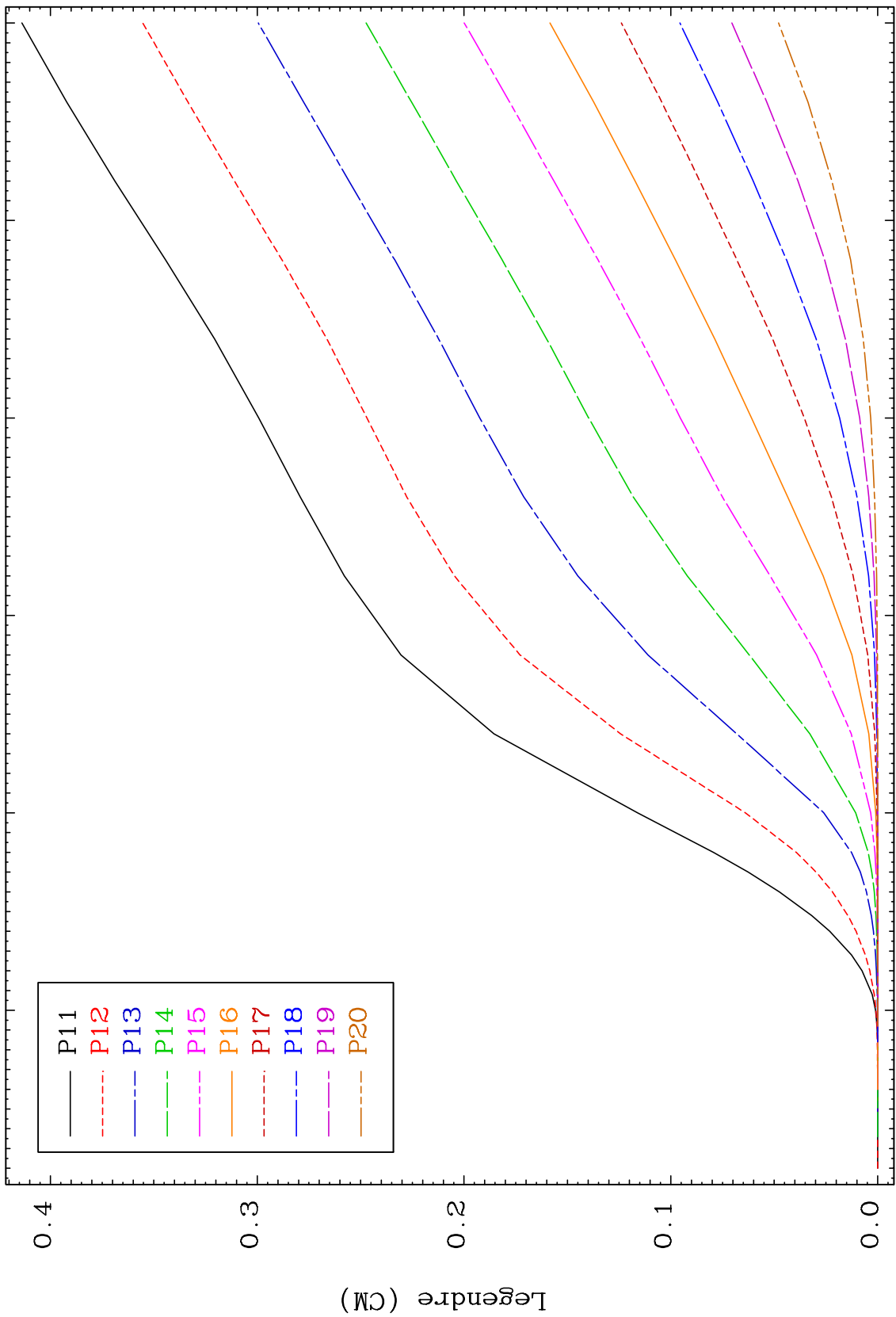
103-Lr-263



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Elastic Legendre Coefficients

103-Lr-263



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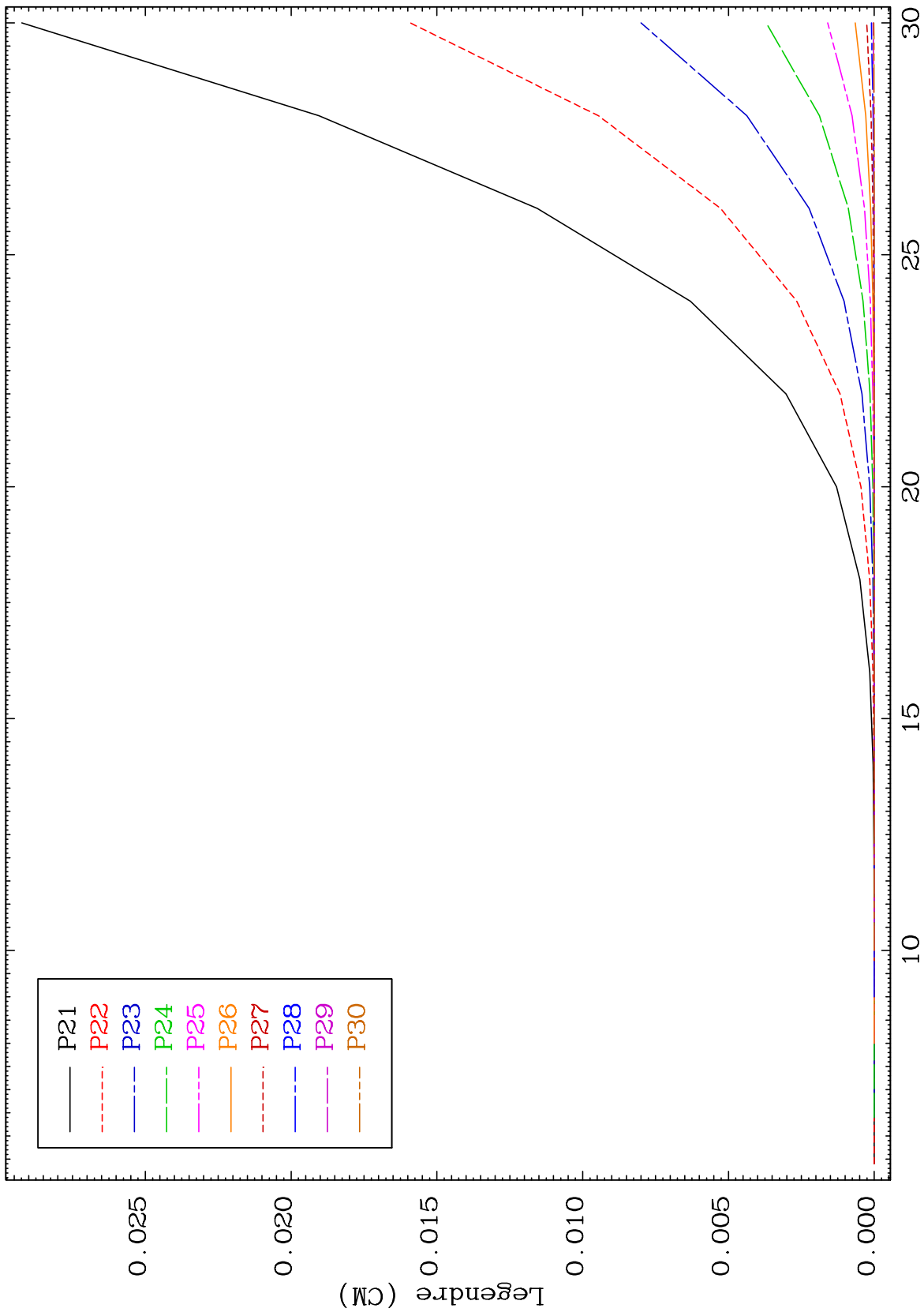
Incident Energy (MeV)

103-Lr-263

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Elastic Legendre Coefficients

103-Lr-263



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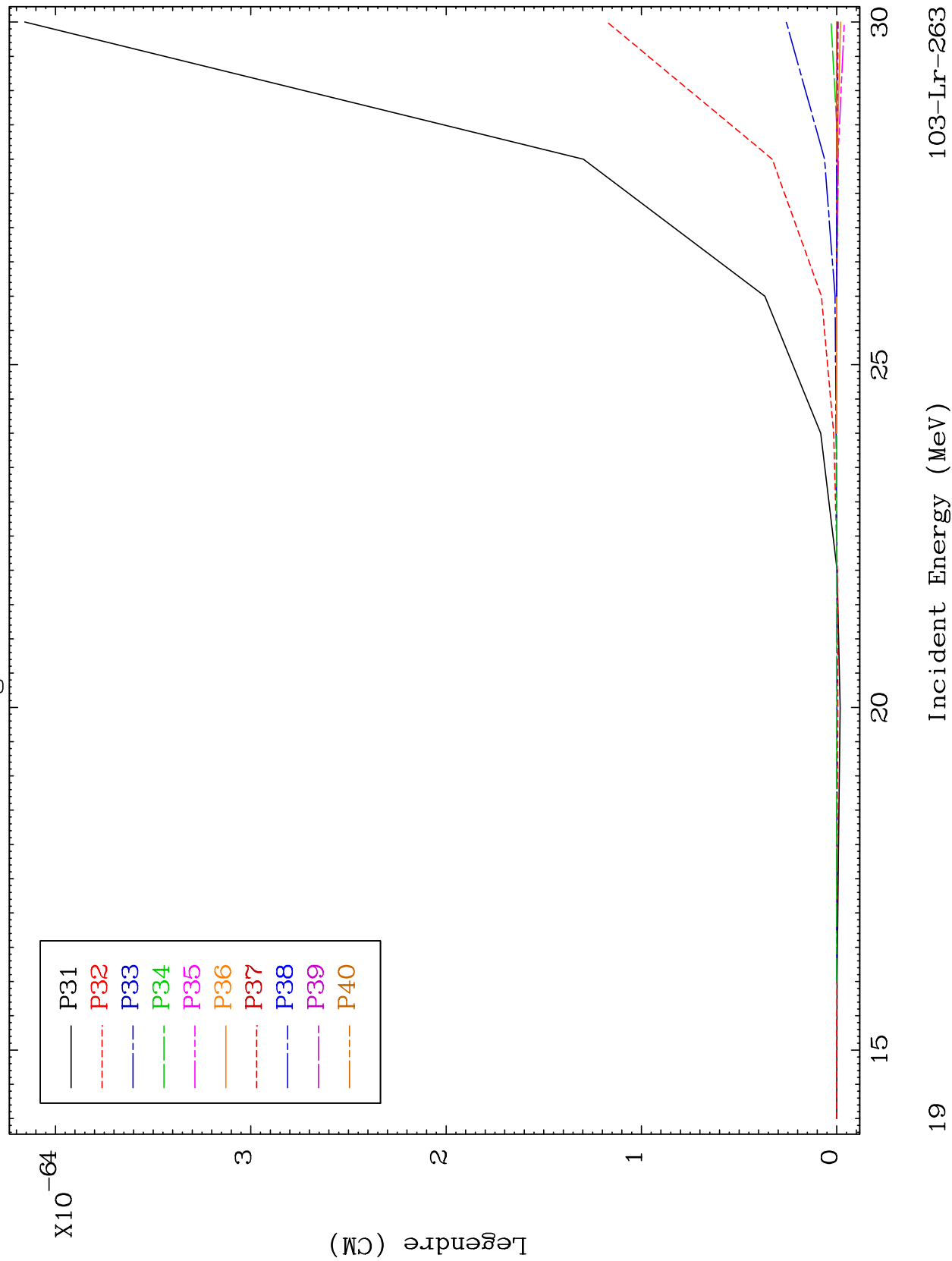
Incident Energy (MeV)

103-Lr-263

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Elastic
Legendre Coefficients

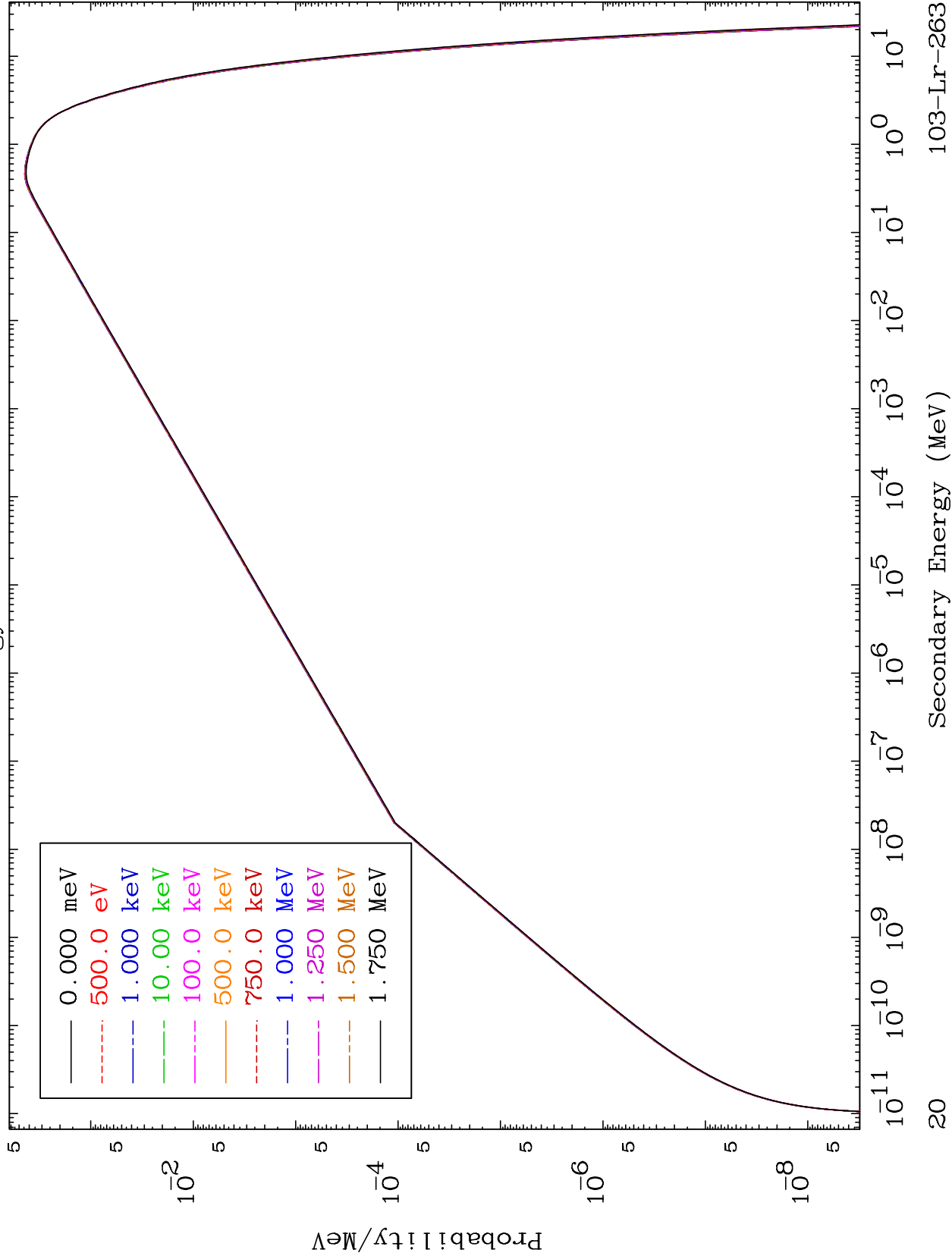
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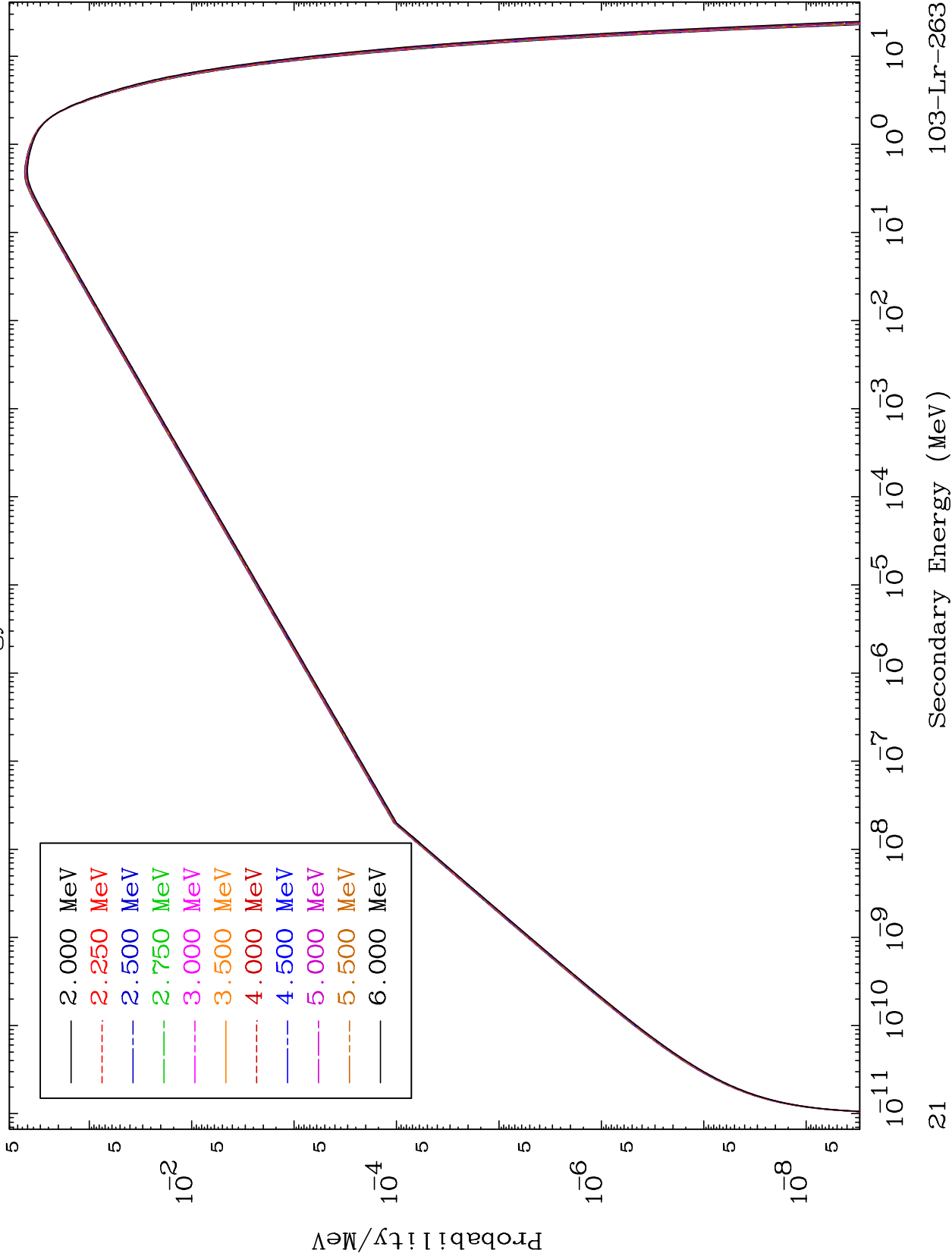
19

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Fission Energy Distributions



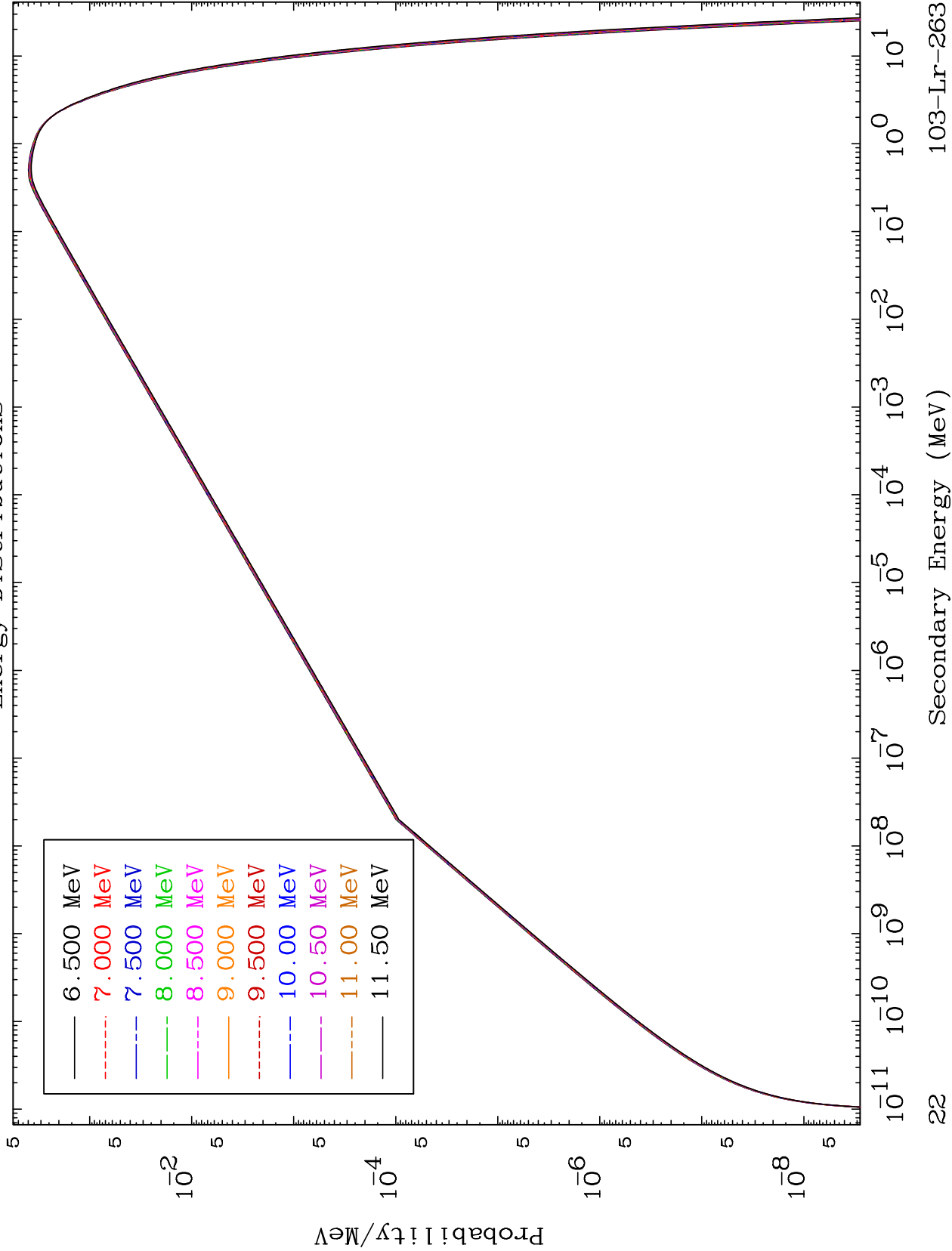
Fission Energy Distributions



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Fission Energy Distributions

103-Lr-263

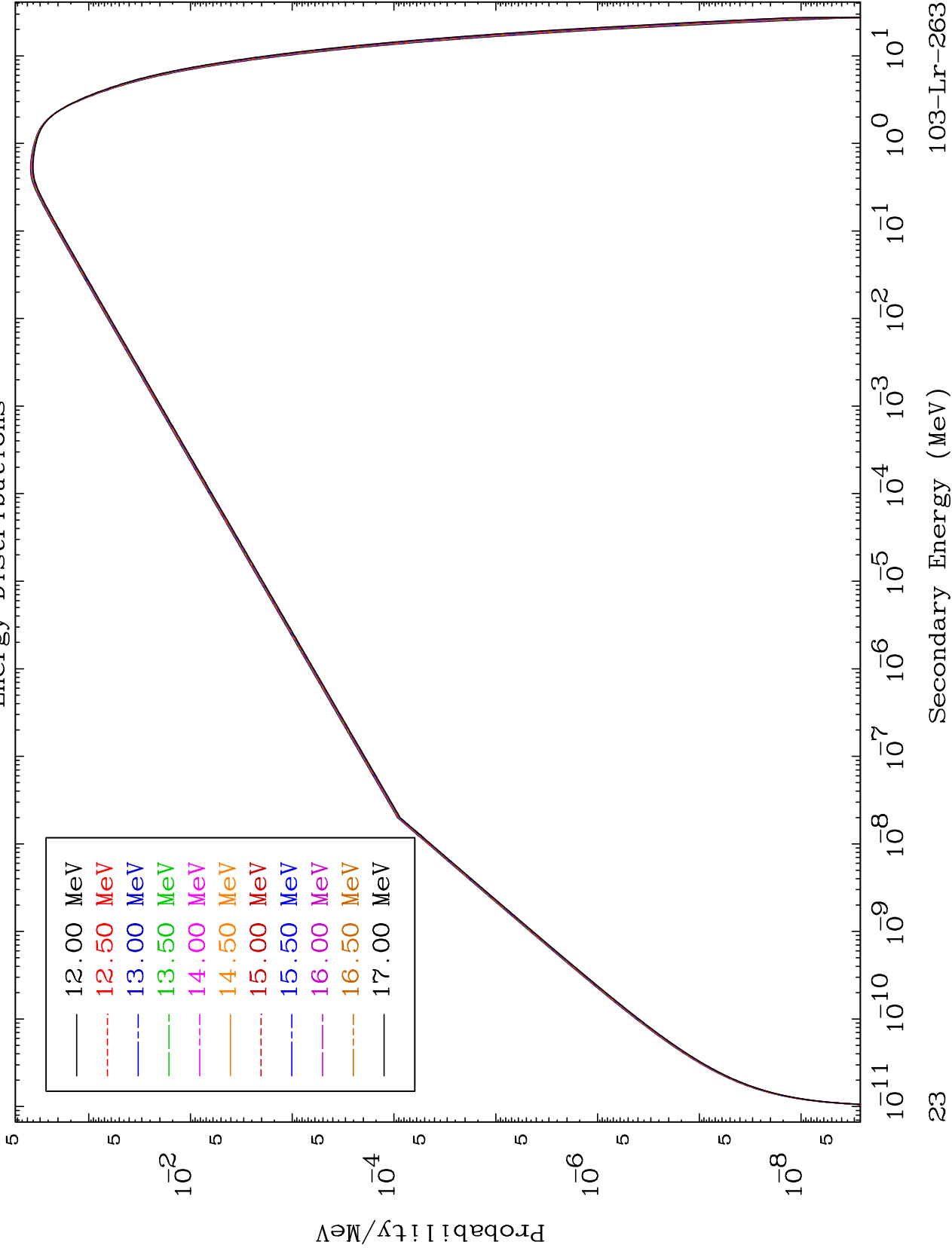


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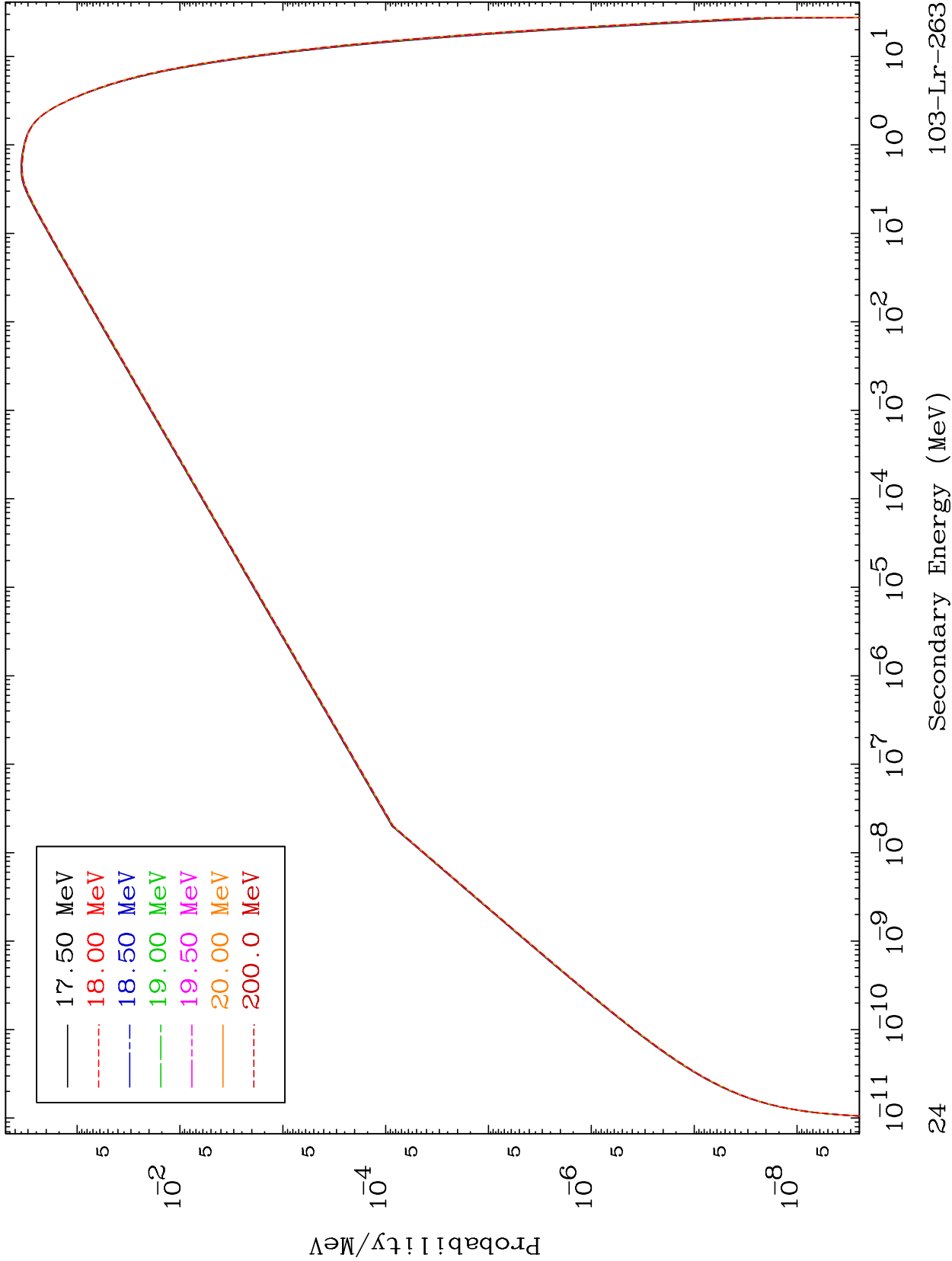
Secondary Energy (MeV)

22

Fission Energy Distributions



Fission Energy Distributions



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$(n,2n) \alpha$

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Radionuclide Production Cross Section

