

Program EVALPLOT
(Version 2021-1)

by

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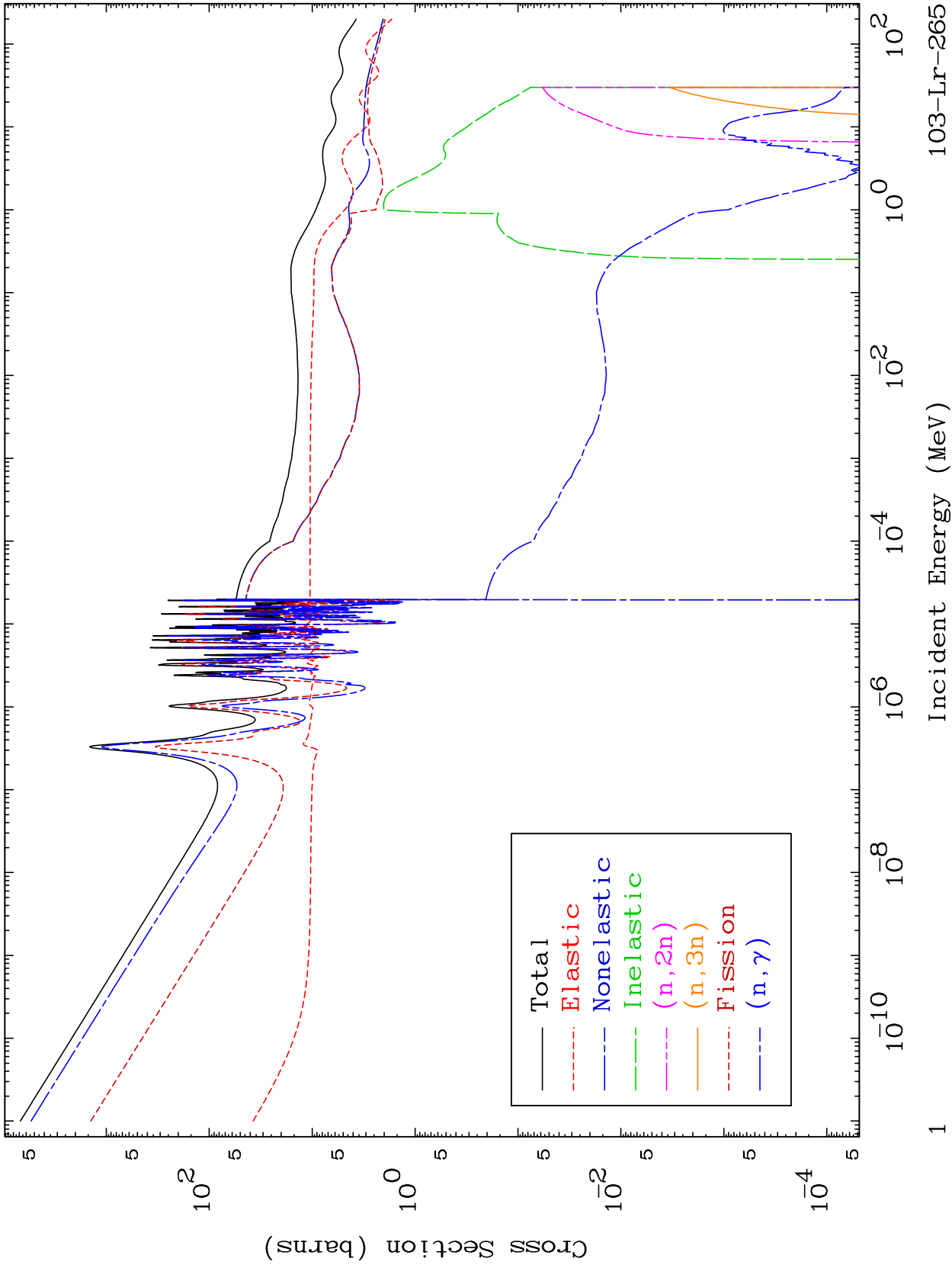
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9994

Neutron Major
293 Kelvin Cross Sections

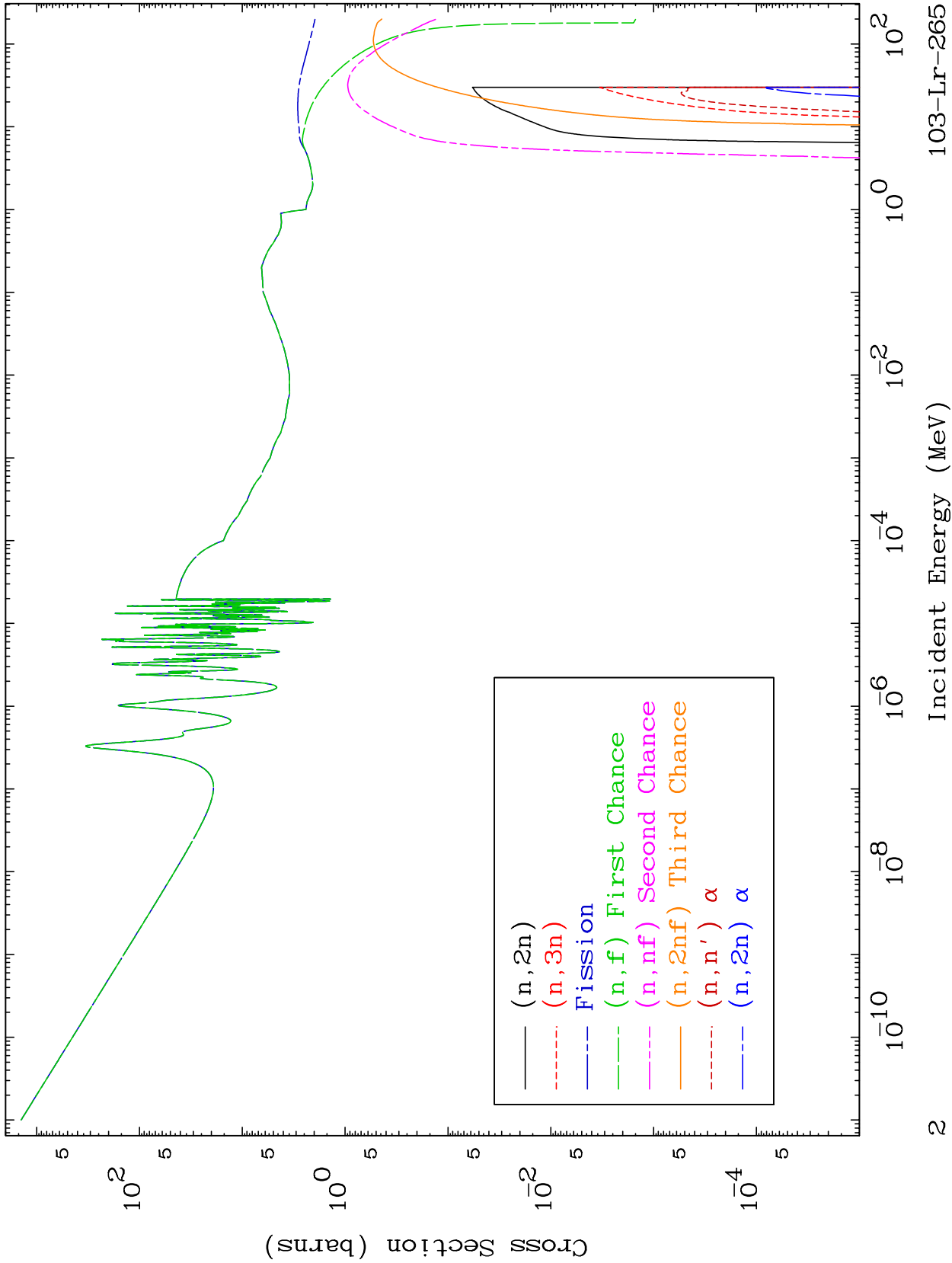
103-Lr-265



MAT 9994

Neutron Absorption
293 Kelvin Cross Sections

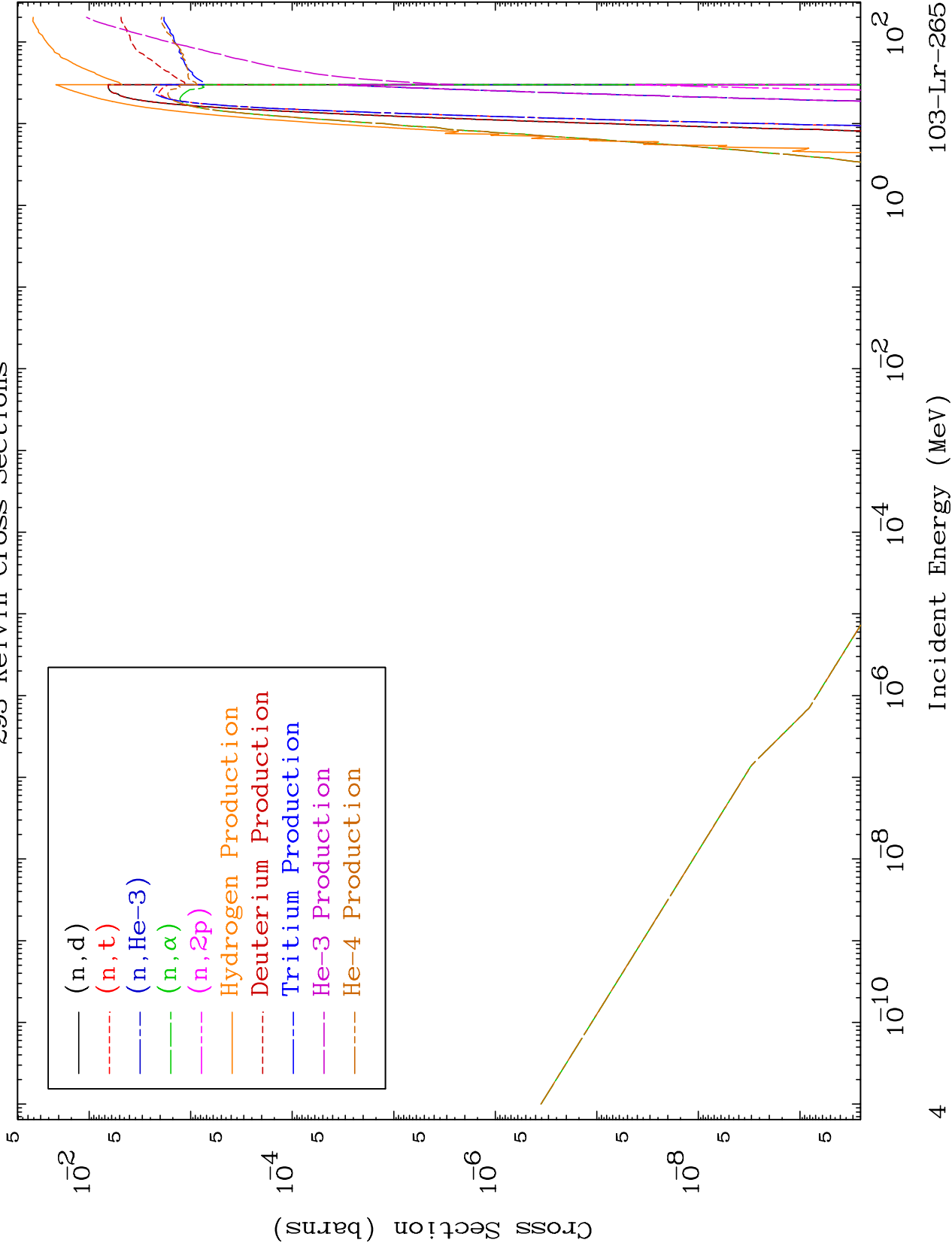
103-Lr-265



MAT 9994

Neutron Absorption
293 Kelvin Cross Sections

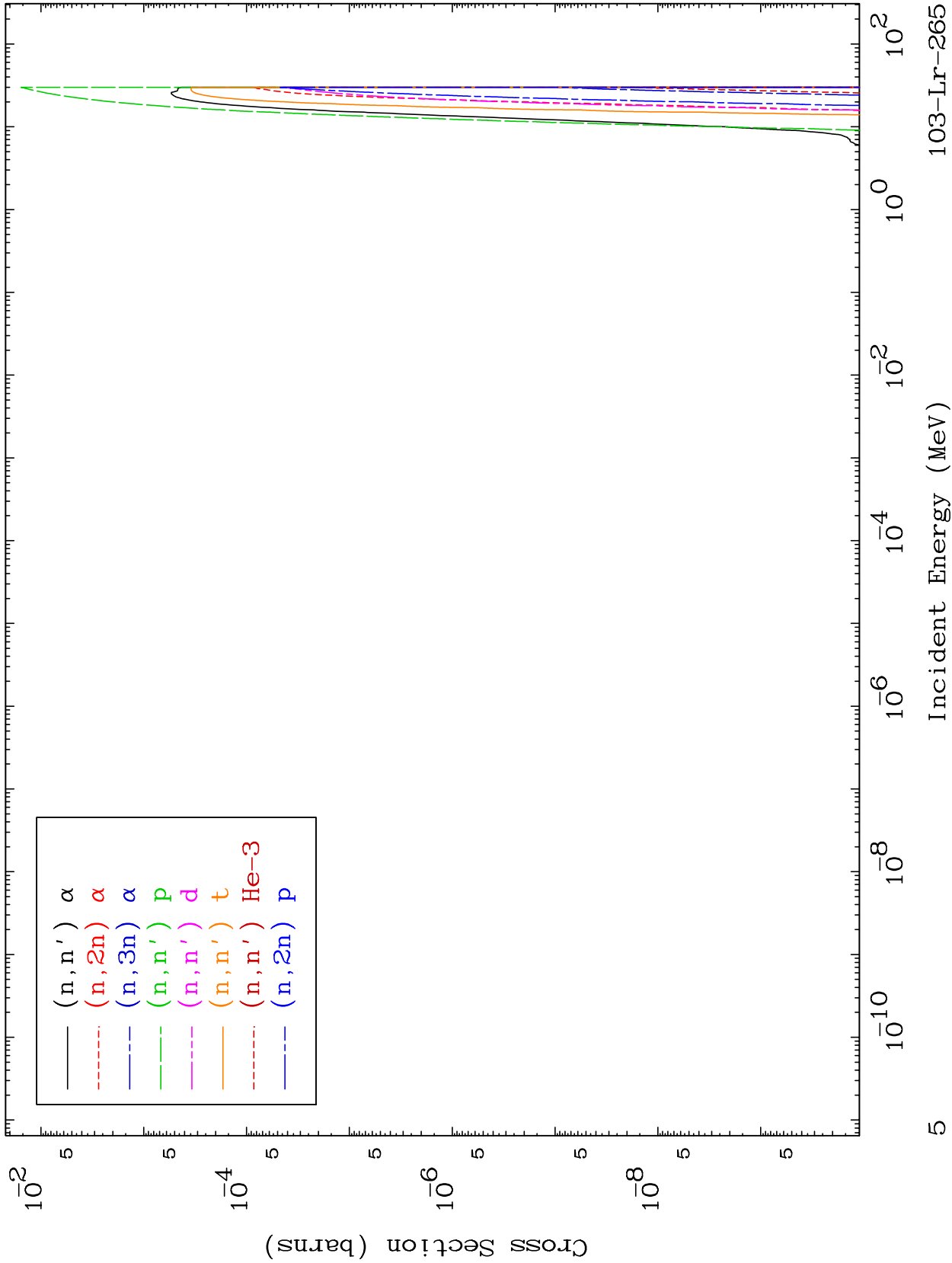
103-Lr-265



MAT 9994

Charged Particle
293 Kelvin Cross Sections

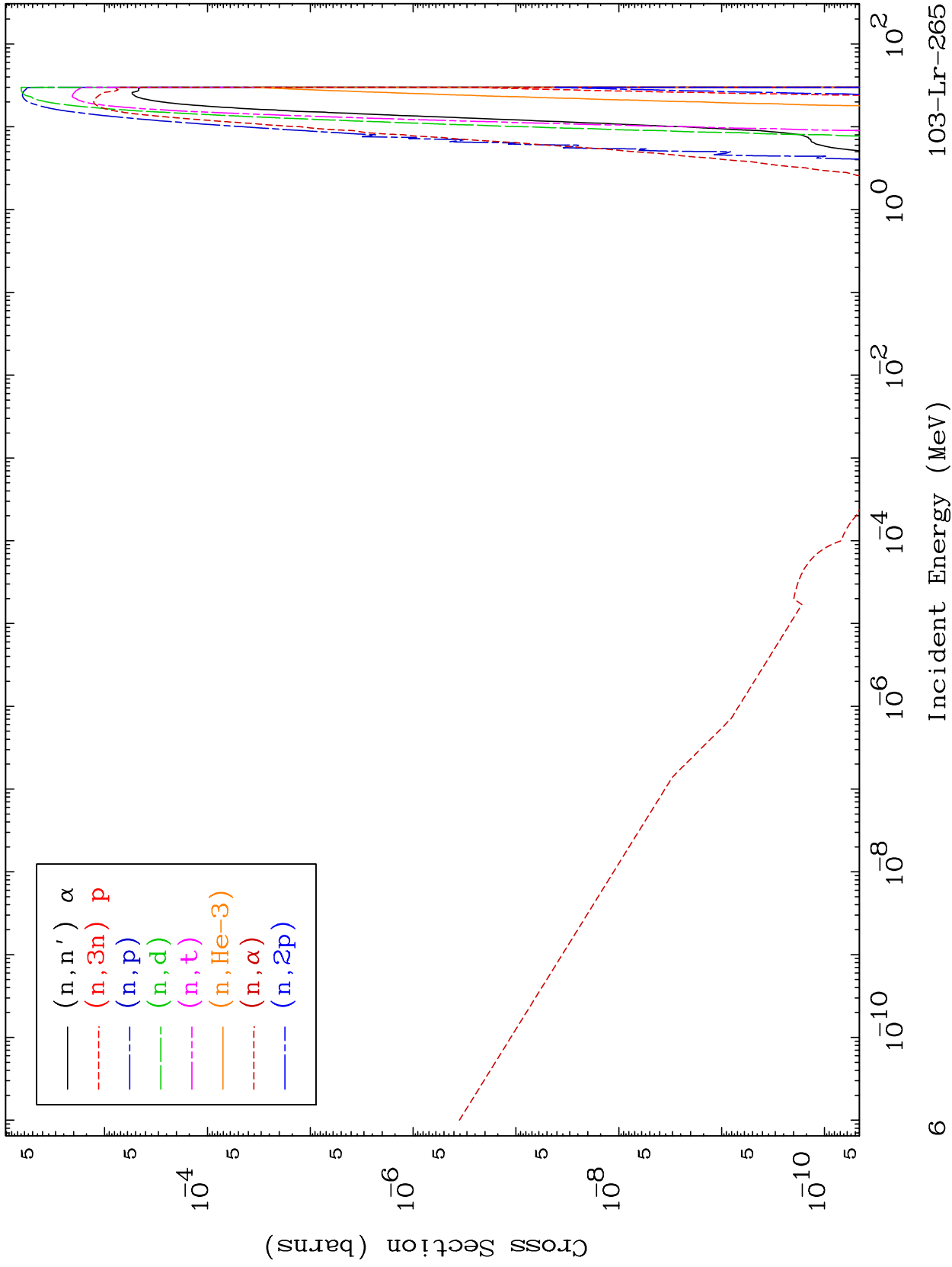
103-Lr-265



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Charged Particle
293 Kelvin Cross Sections

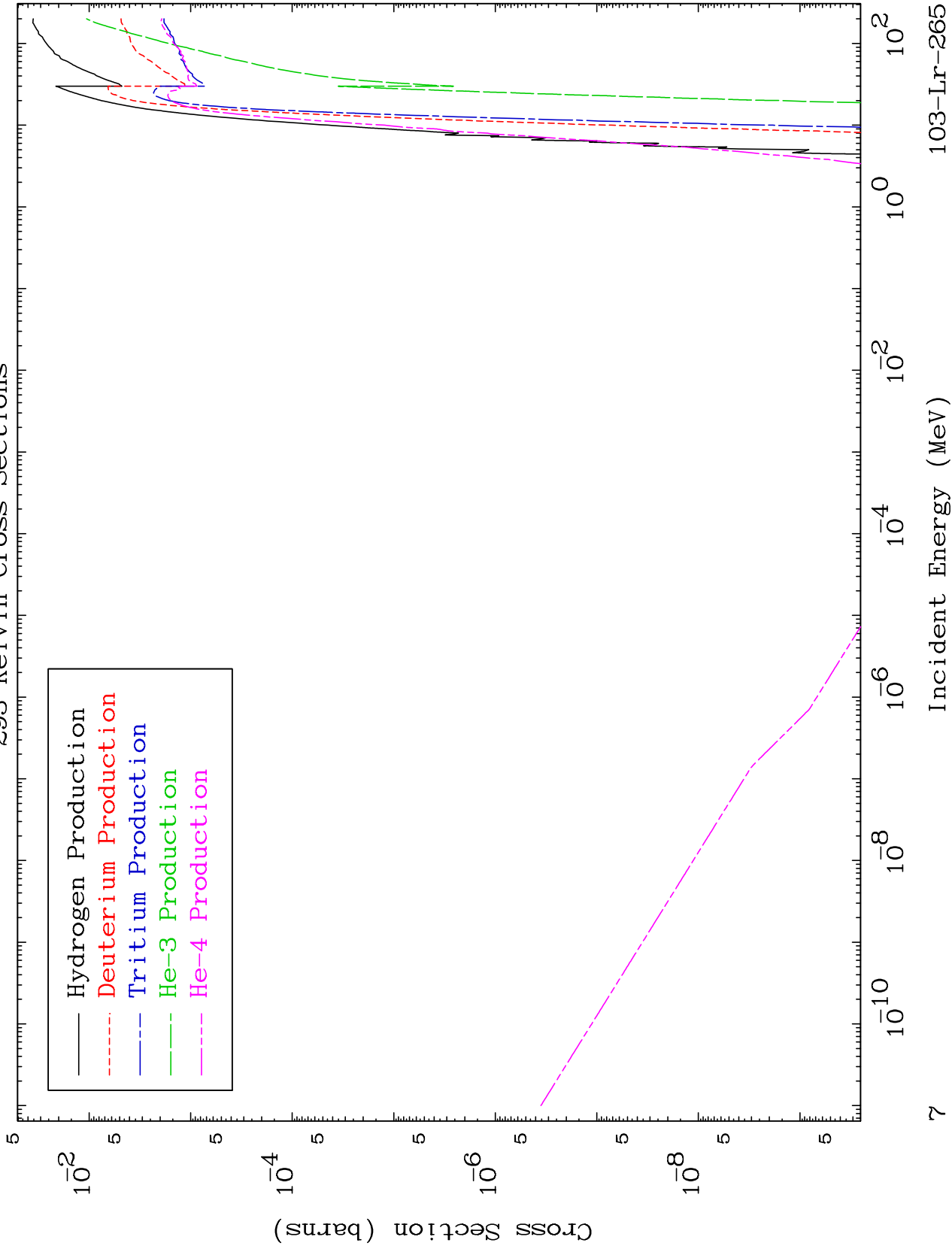
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Particle Production
293 Kelvin Cross Sections

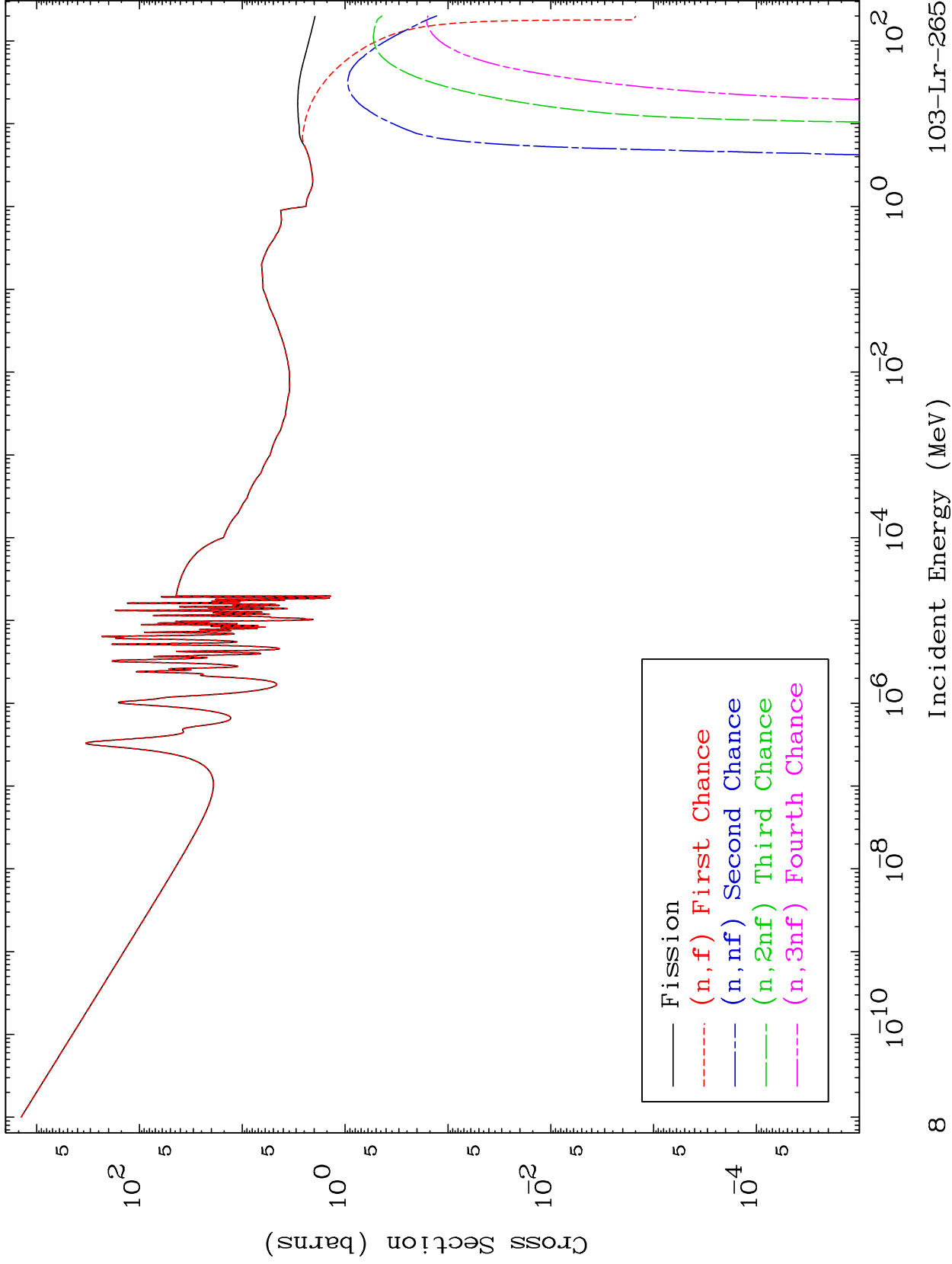
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Fission
293 Kelvin Cross Sections

103-Lr-265

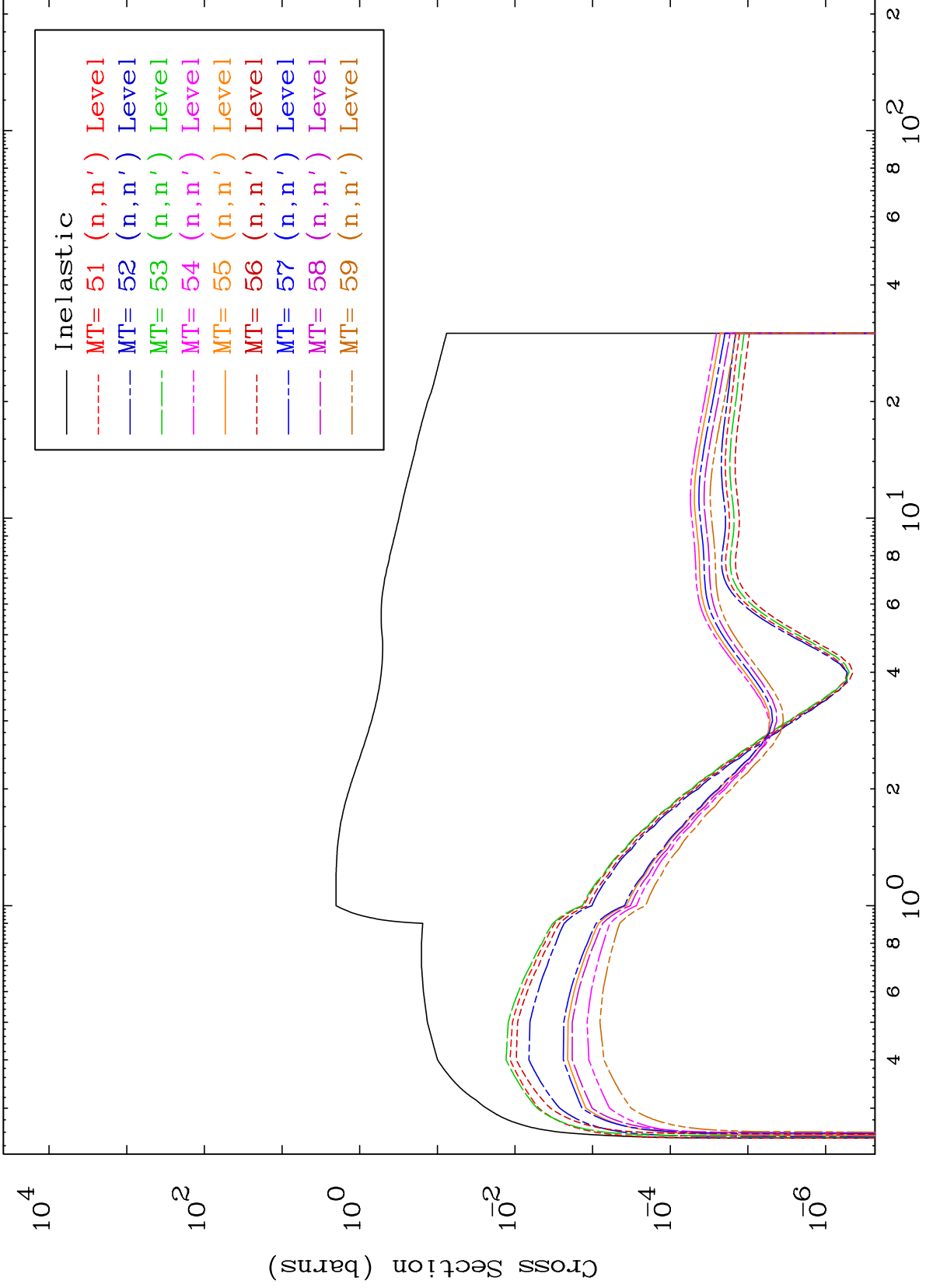


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(n,n') Levels

103-Lr-265

293 Kelvin Cross Sections

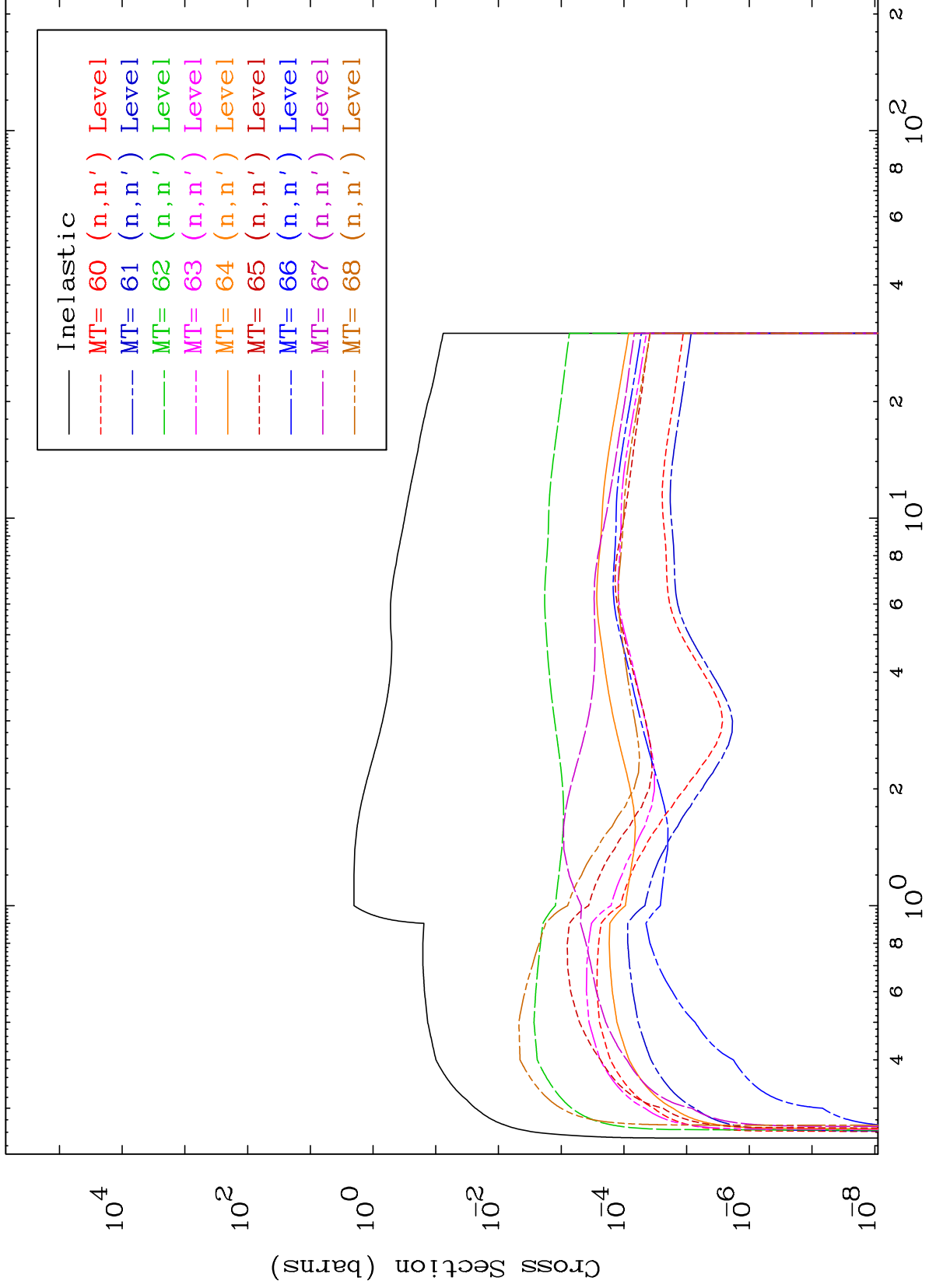


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(n,n') Levels

103-Lr-265

293 Kelvin Cross Sections



10

Incident Energy (MeV)

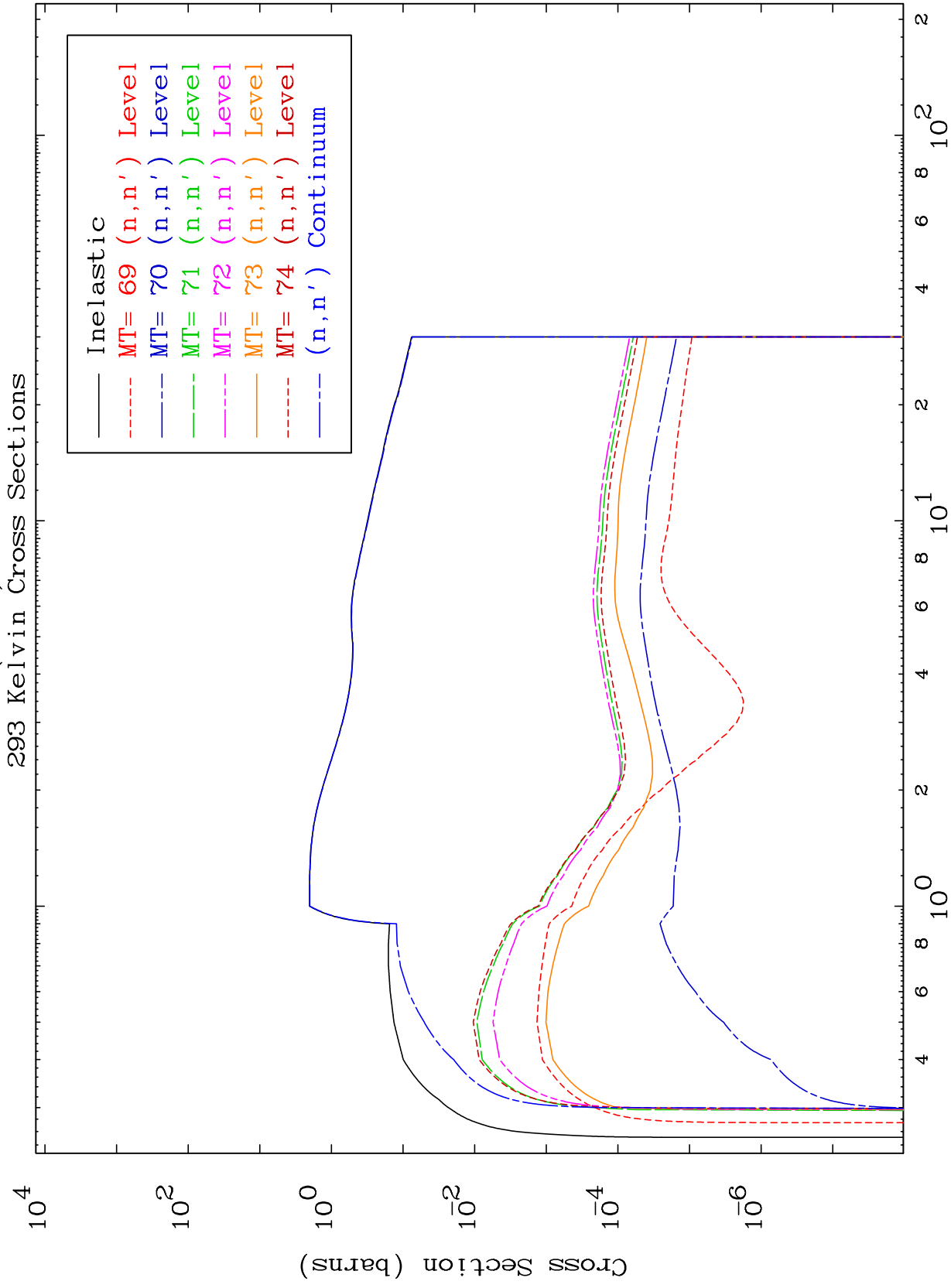
103-Lr-265

MAT 9994

(n,n') Levels

103-Lr-265

293 Kelvin Cross Sections



11

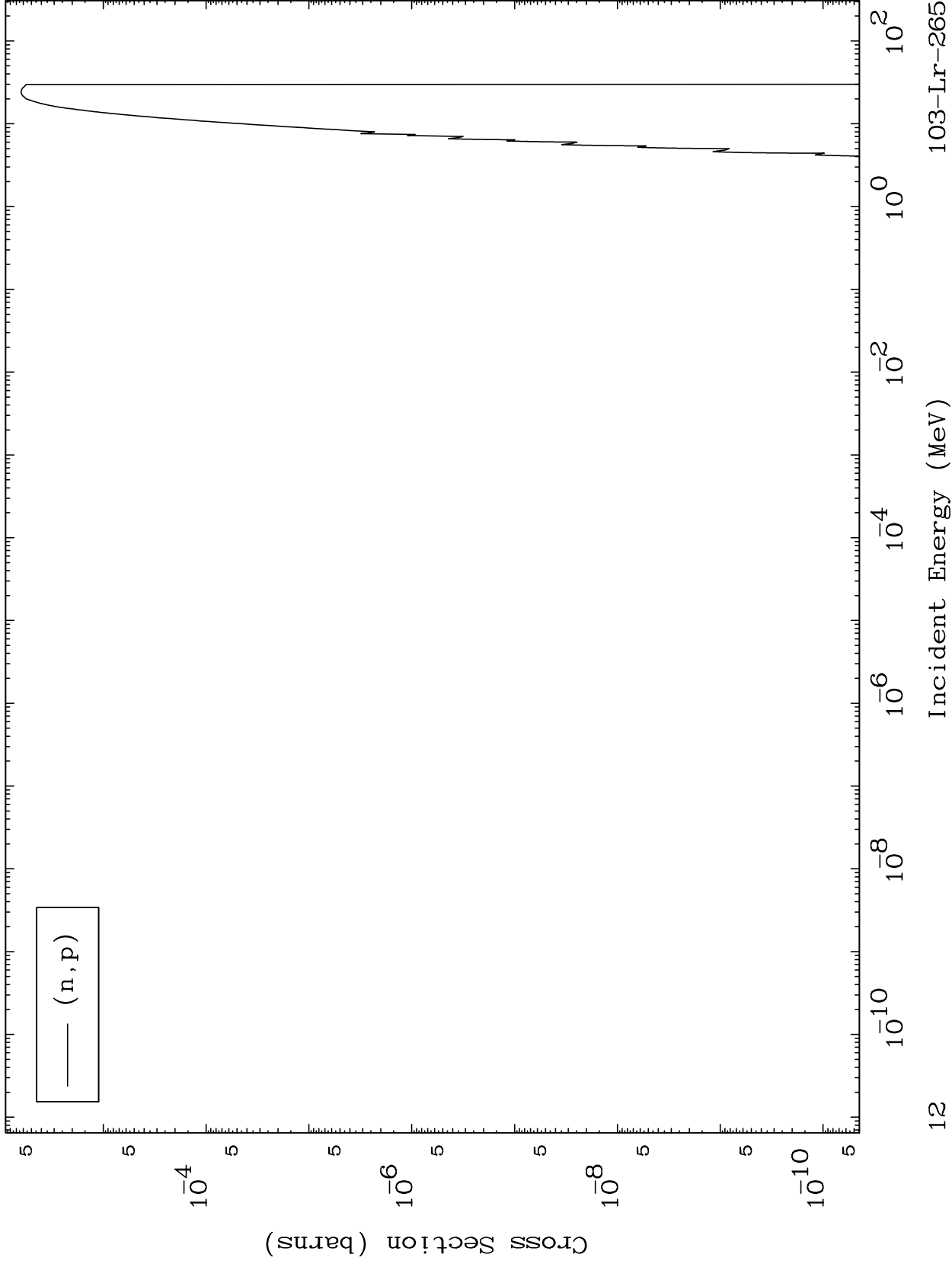
Incident Energy (MeV)

103-Lr-265

MAT 9994

(n,p) Levels
293 Kelvin Cross Sections

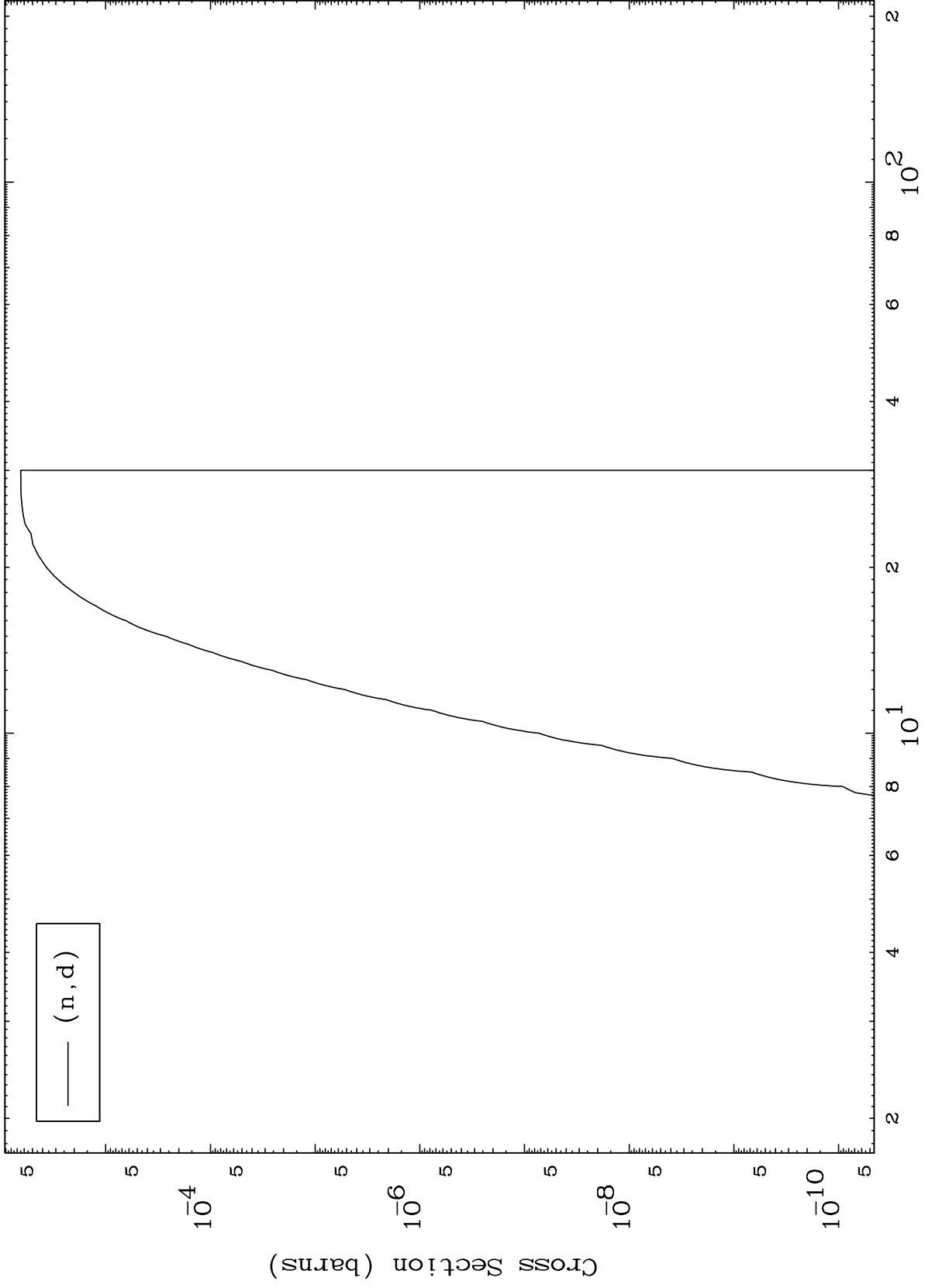
103-Lr-265



MAT 9994

(n,d) Levels
293 Kelvin Cross Sections

103-Lr-265



13

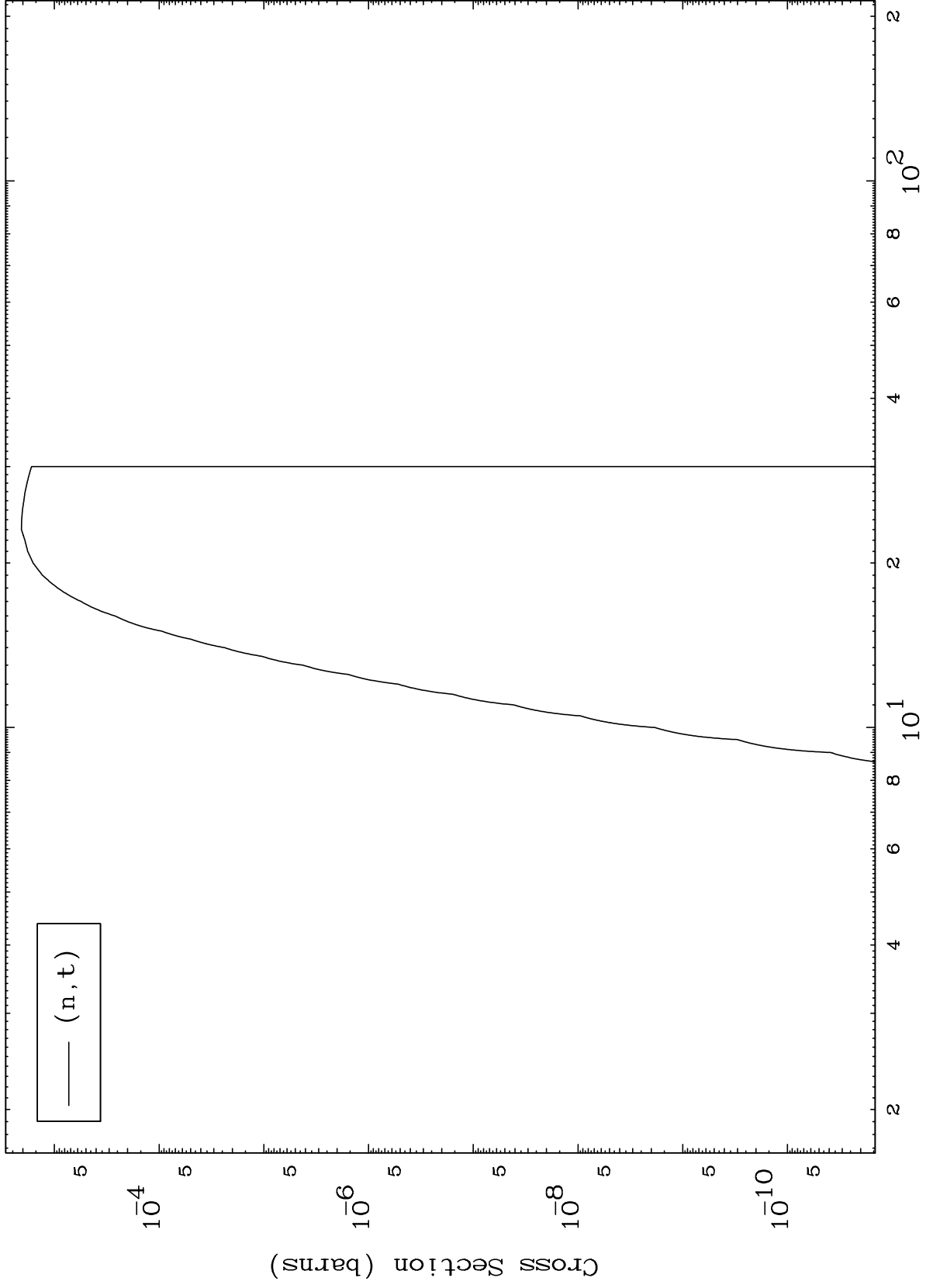
Incident Energy (MeV)

103-Lr-265

MAT 9994

(n,t) Levels
293 Kelvin Cross Sections

103-Lr-265



14

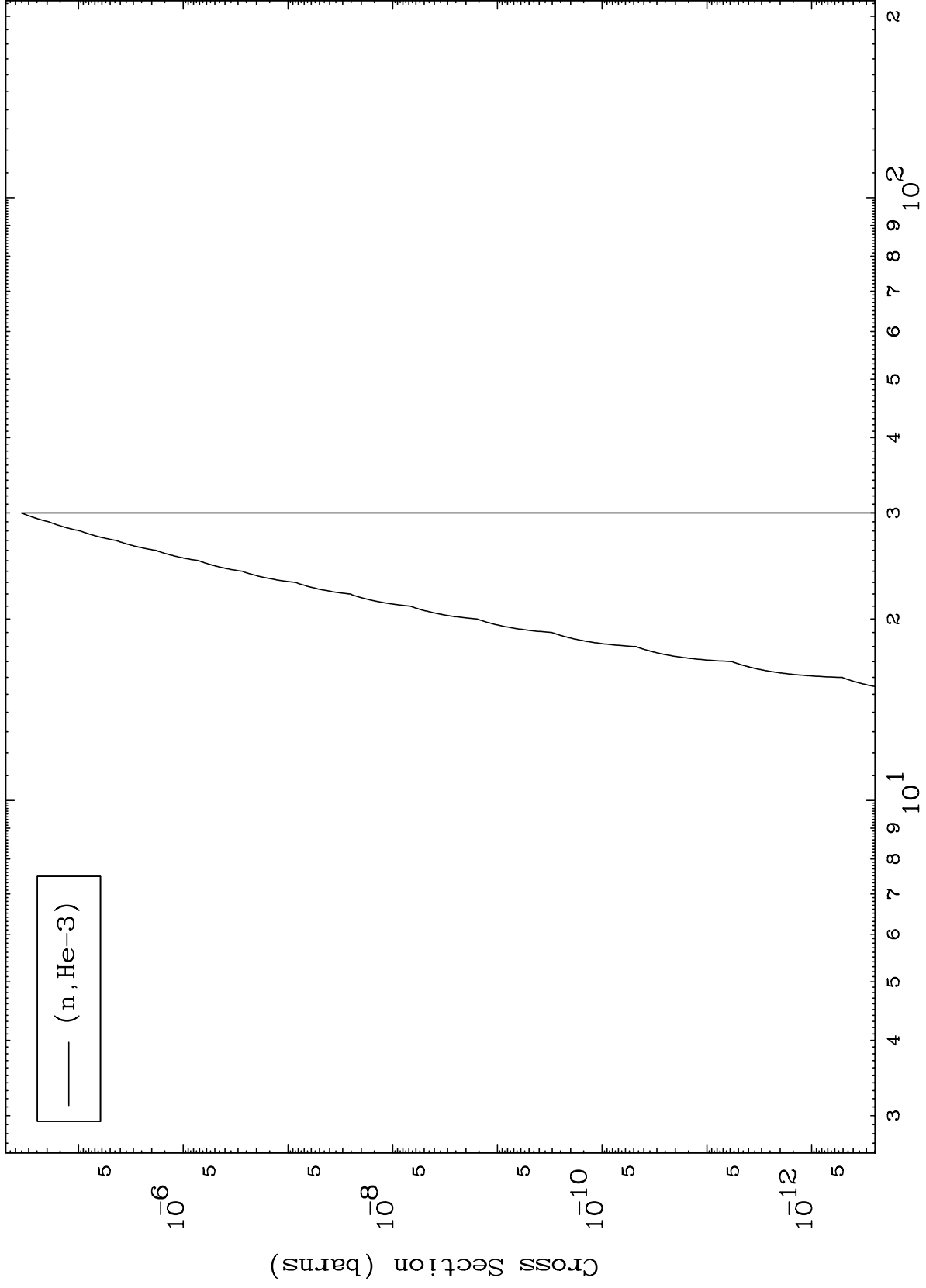
Incident Energy (MeV)

103-Lr-265

MAT 9994

(n,He3) Levels
293 Kelvin Cross Sections

103-Lr-265



15

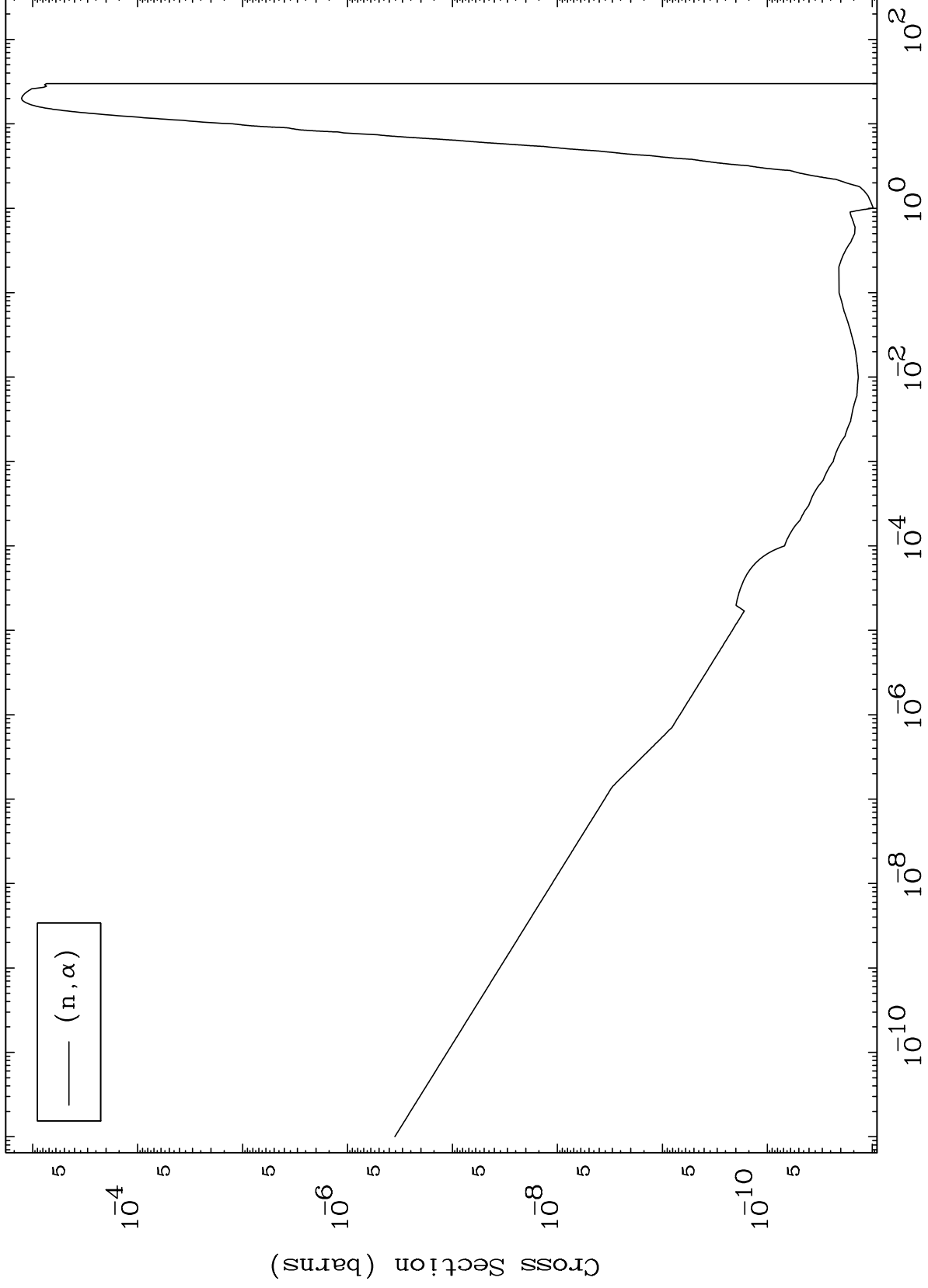
Incident Energy (MeV)

103-Lr-265

MAT 9994

(n, α) Levels
293 Kelvin Cross Sections

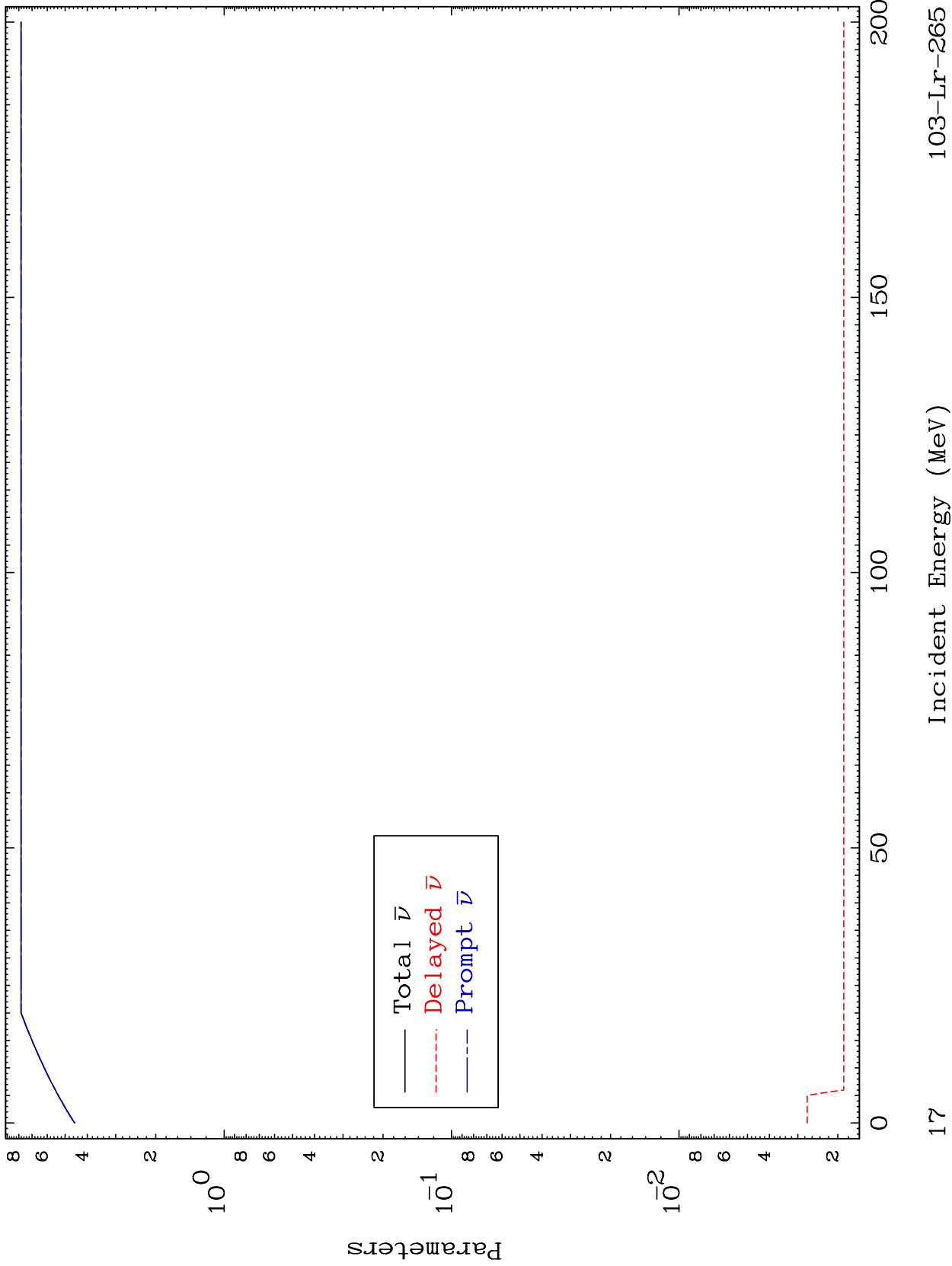
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Energy Release
Parameters

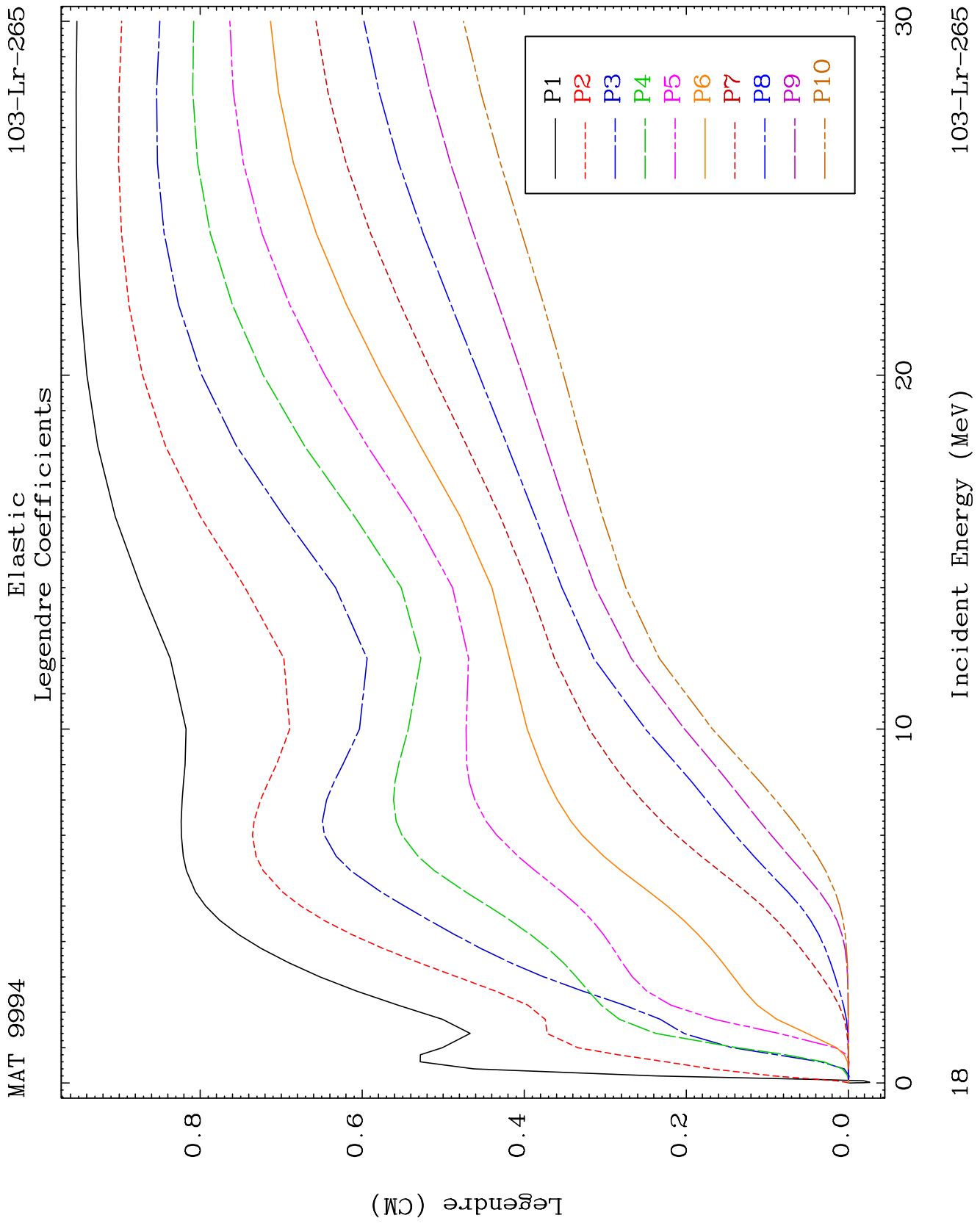
103-Lr-265



17

Incident Energy (MeV)

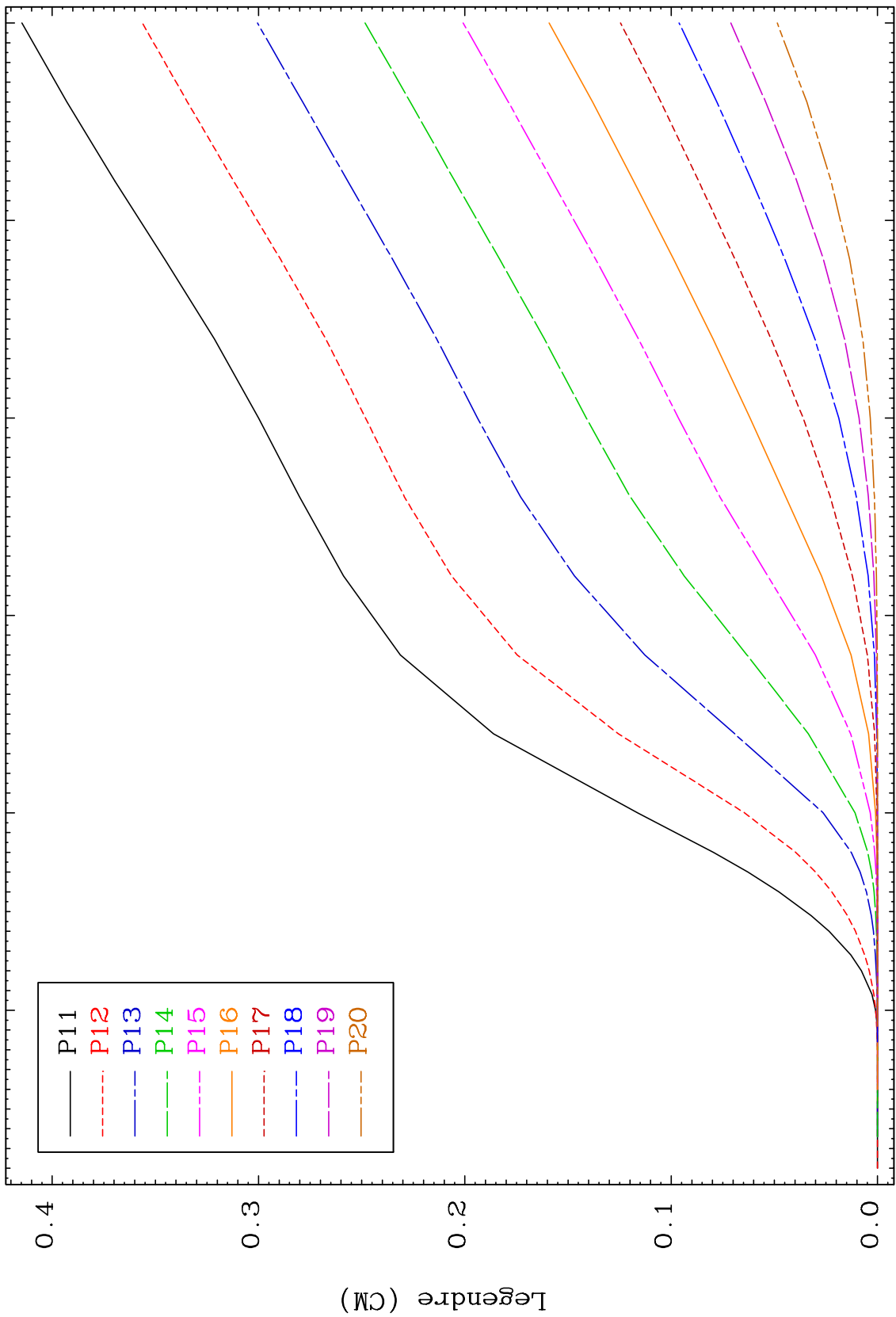
103-Lr-265



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Elastic Legendre Coefficients

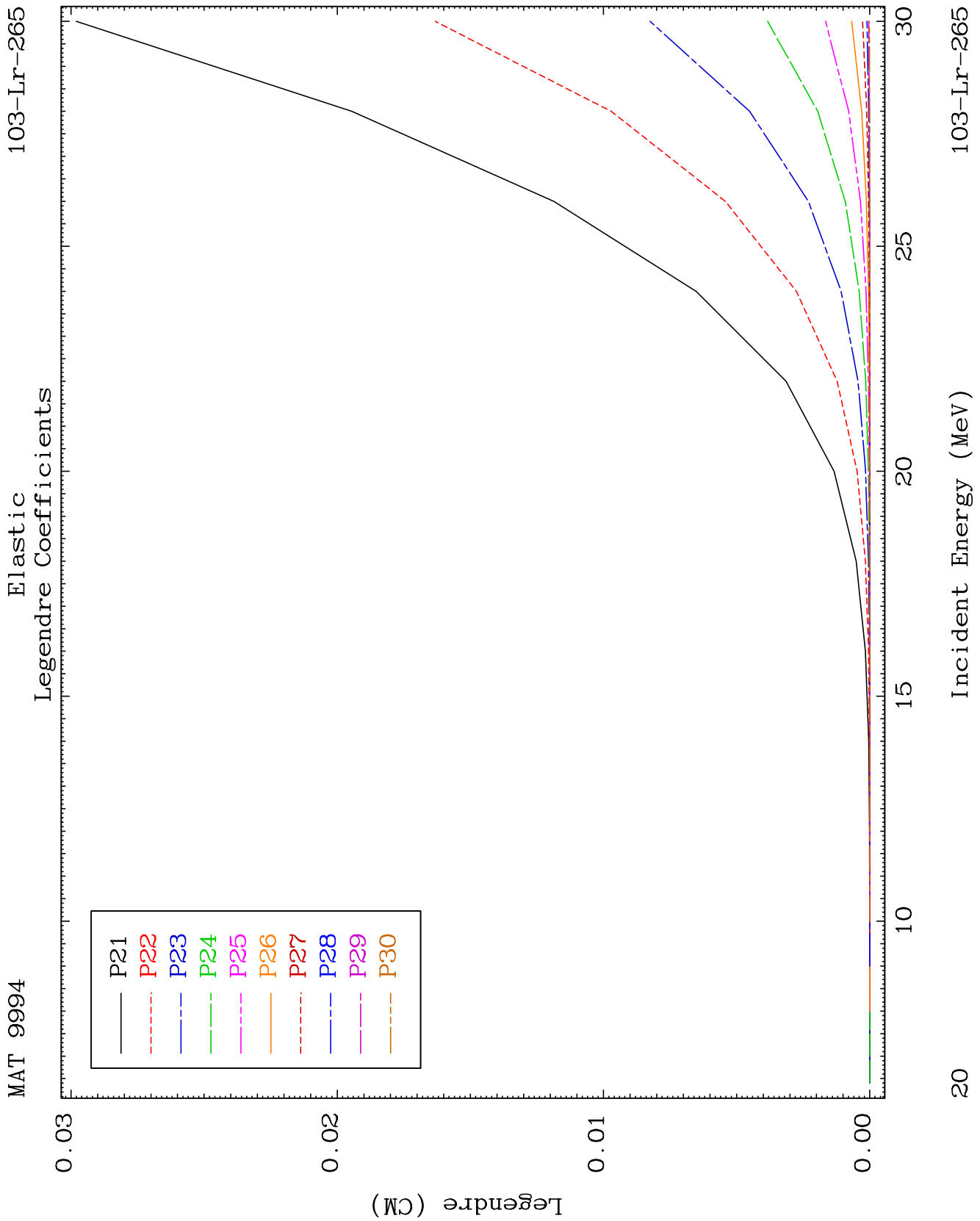
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19

Incident Energy (MeV)

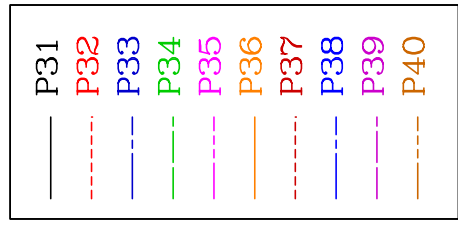
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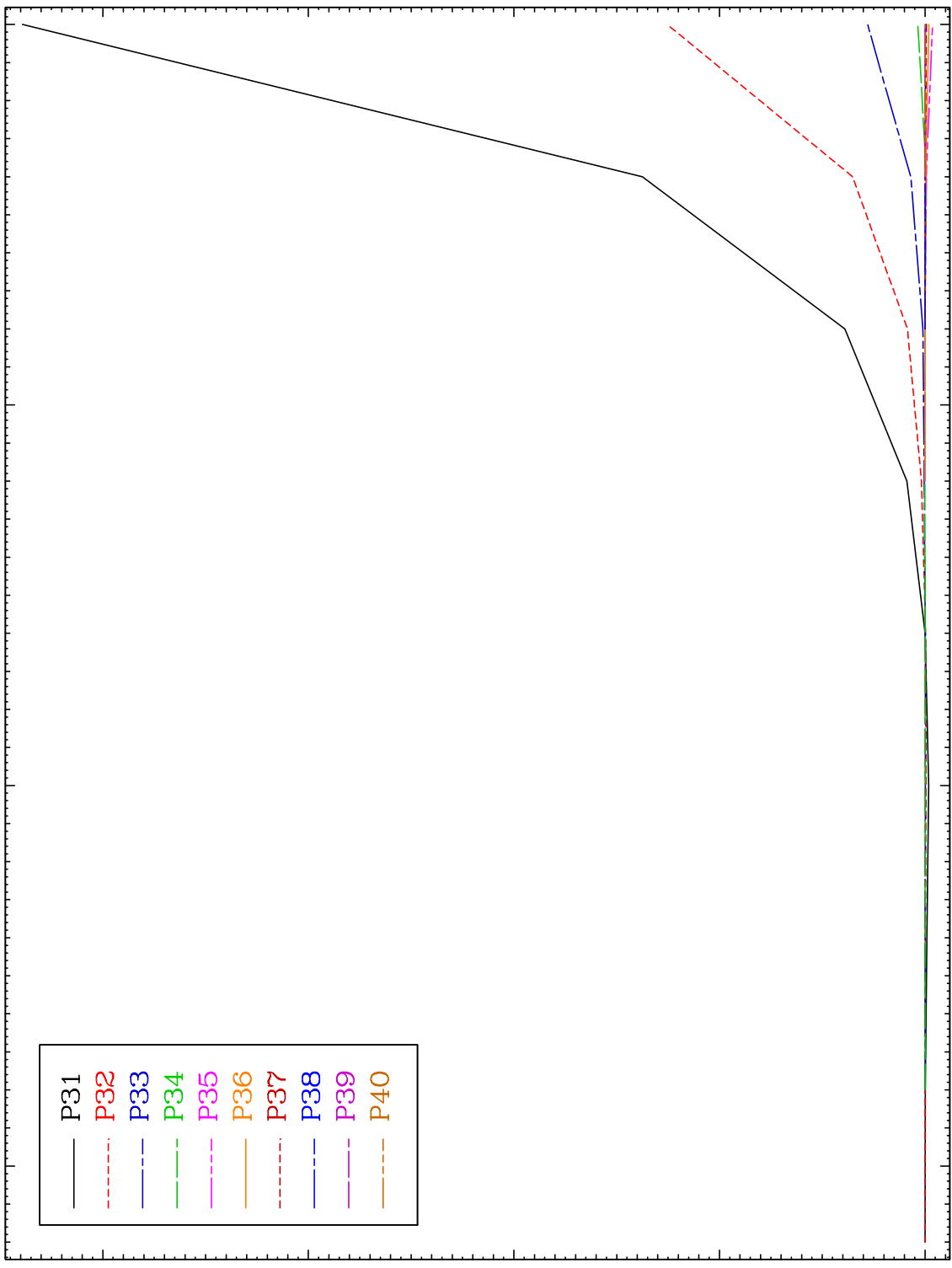
Elastic Legendre Coefficients

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$\times 10^{-6}$

Legendre (CM)



30

25

20

15

21

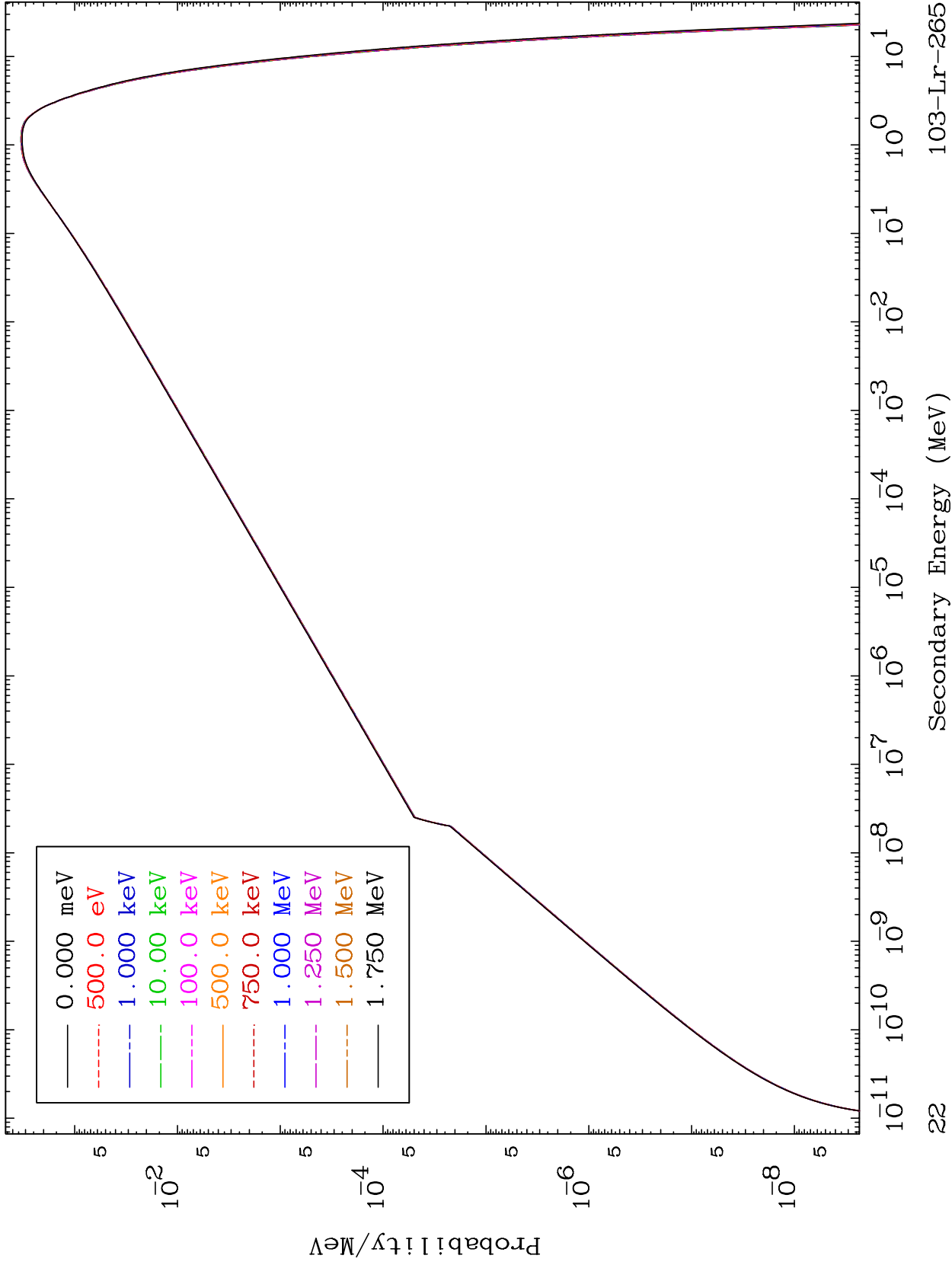
Incident Energy (MeV)

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Fission Energy Distributions

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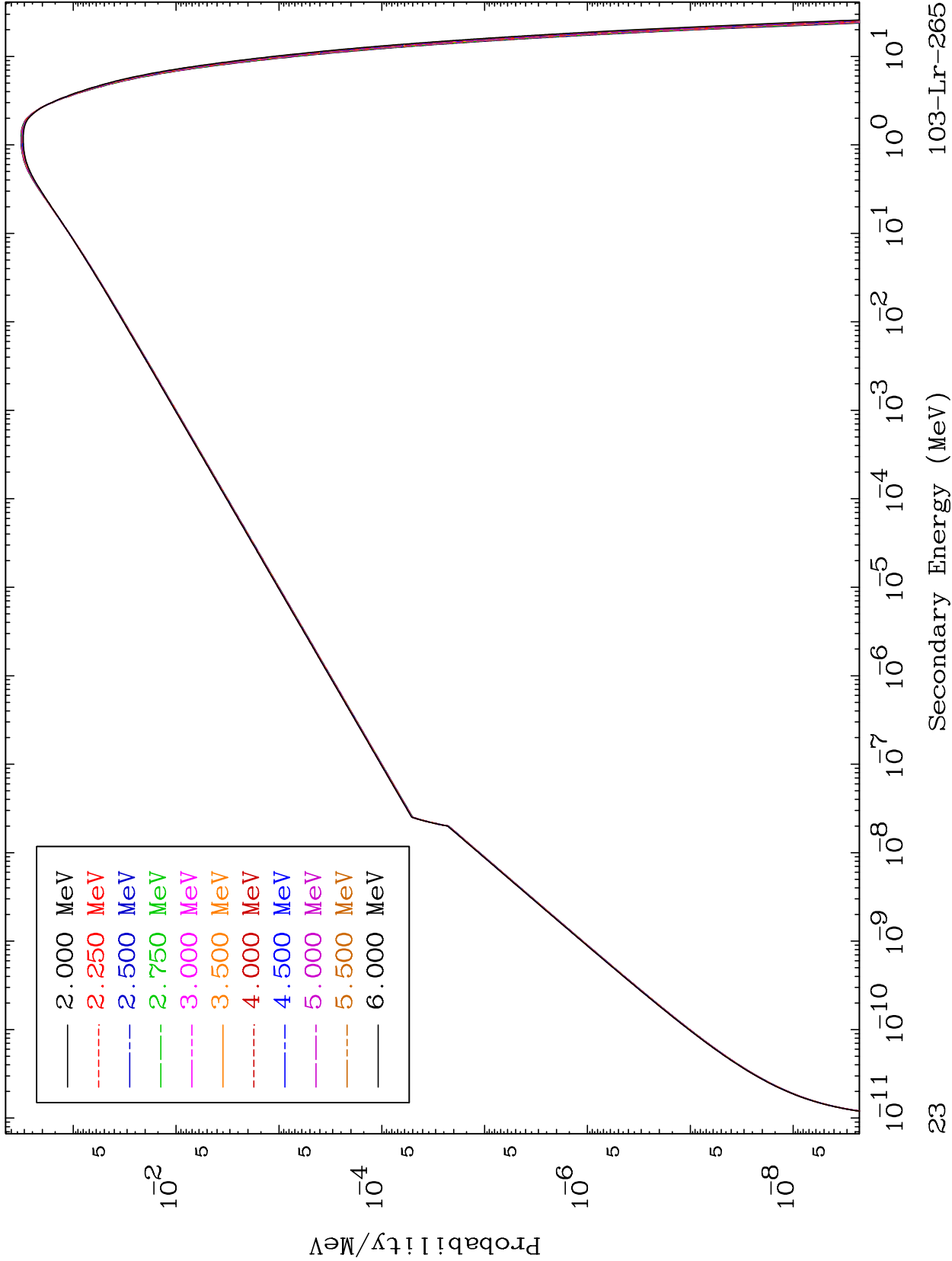


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Fission Energy Distributions

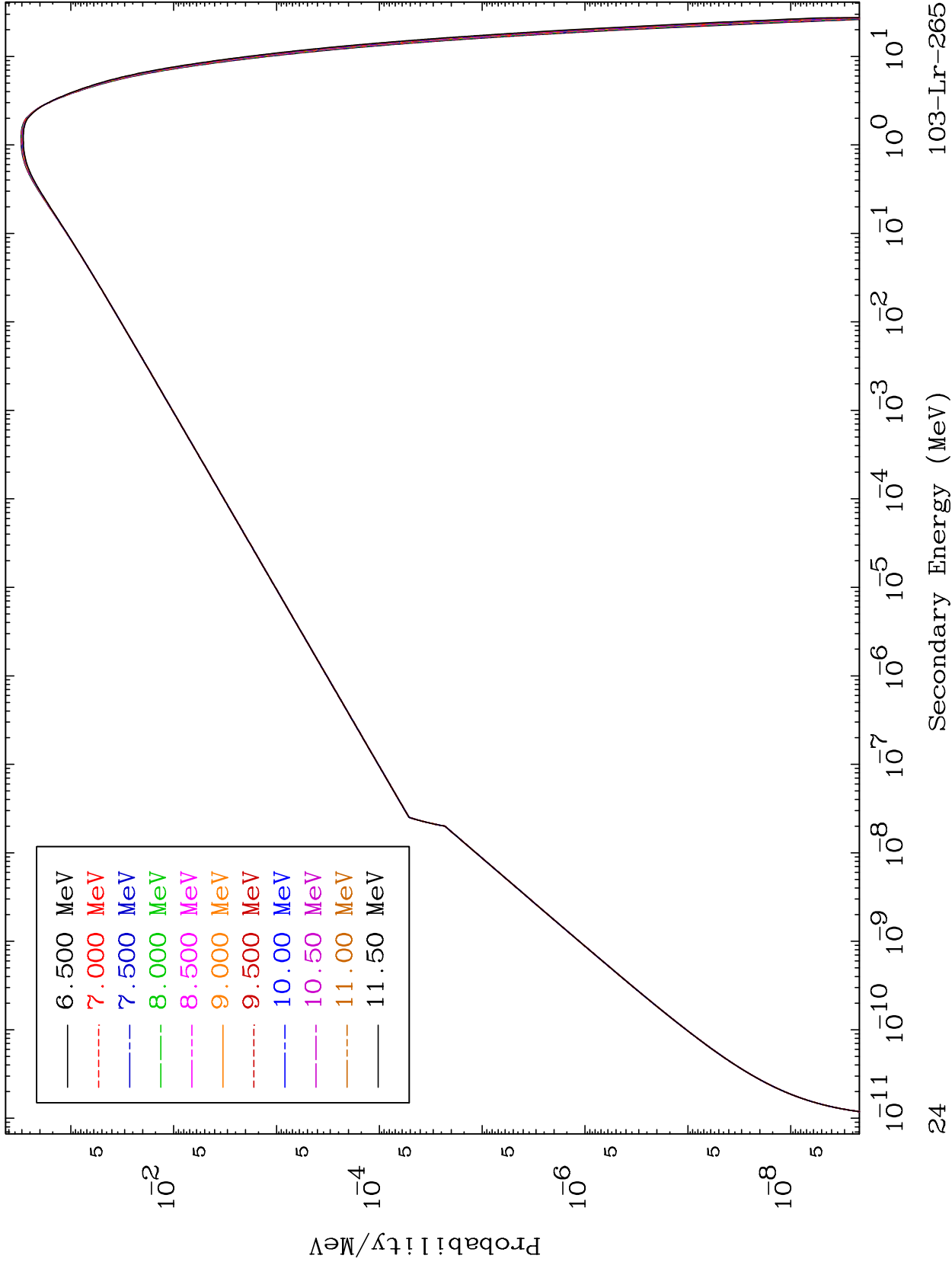
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Fission Energy Distributions

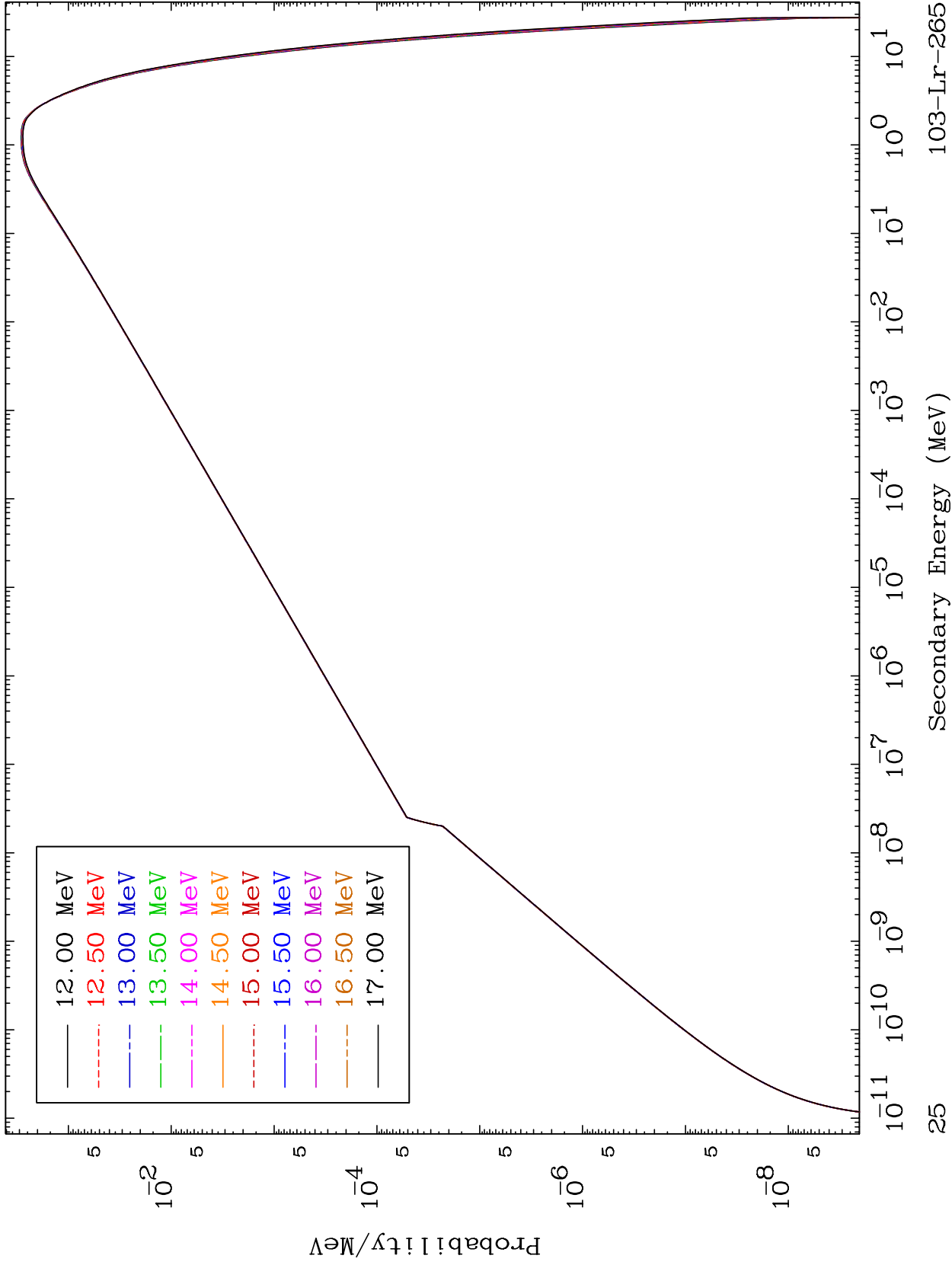
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Fission
Energy Distributions

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MAT 9994

Fission
Energy Distributions

103-Lr-265

