

Program EVALPLOT
(Version 2021-1)

by

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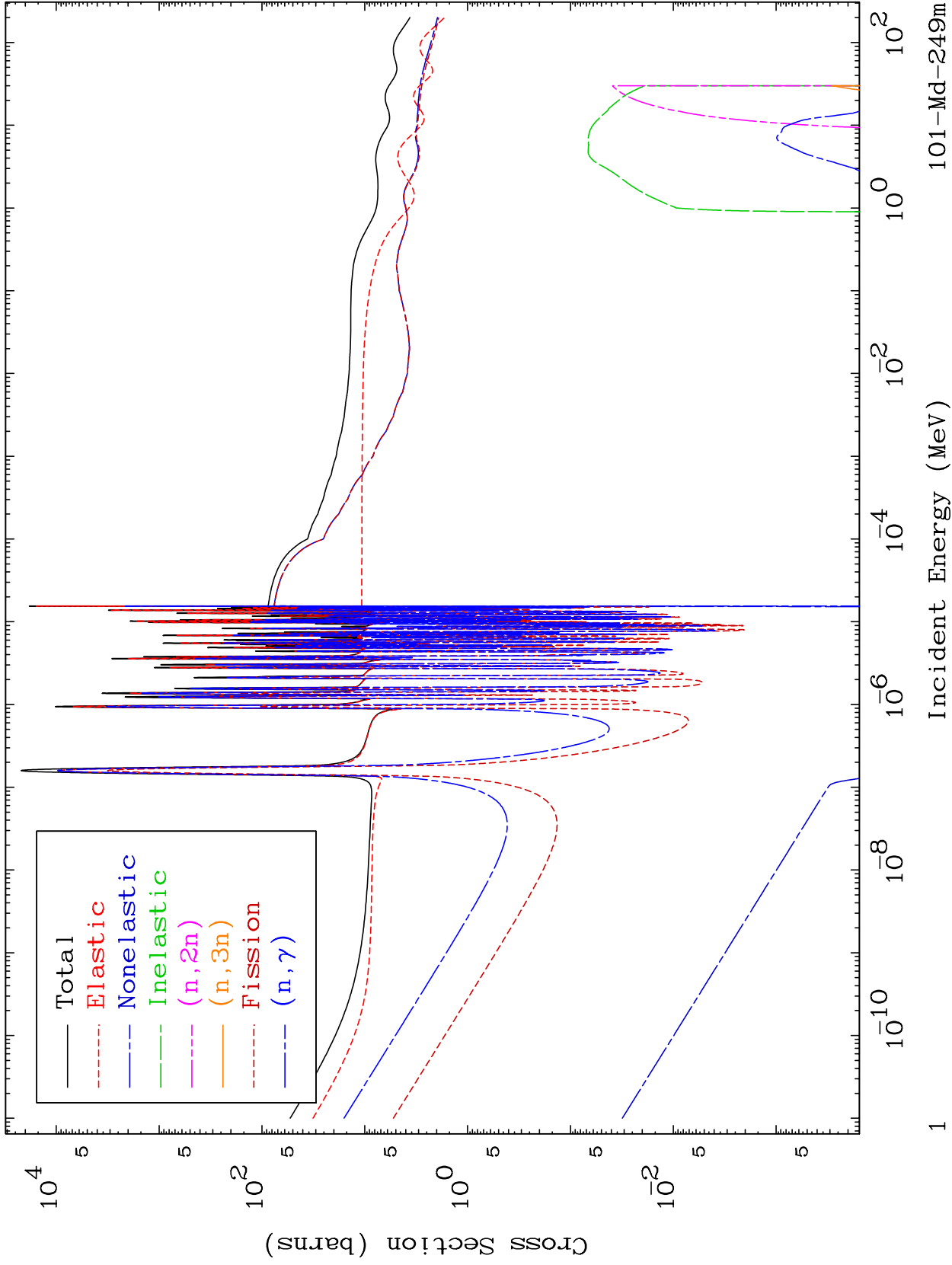
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Press Mouse Button to Start

MAT 9955

Neutron Major
293 Kelvin Cross Sections

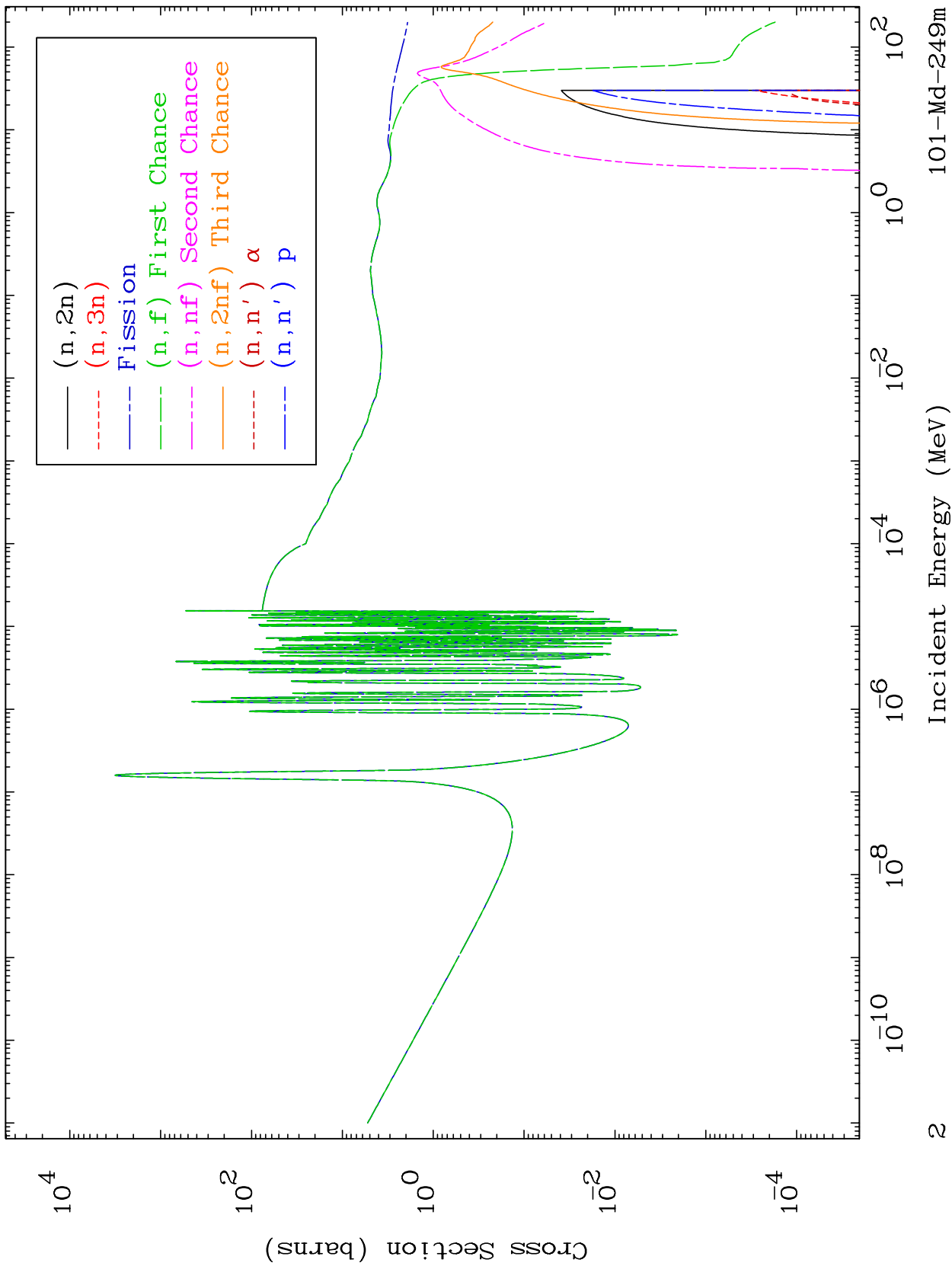
101-Md-249m



MAT 9955

Neutron Absorption
293 Kelvin Cross Sections

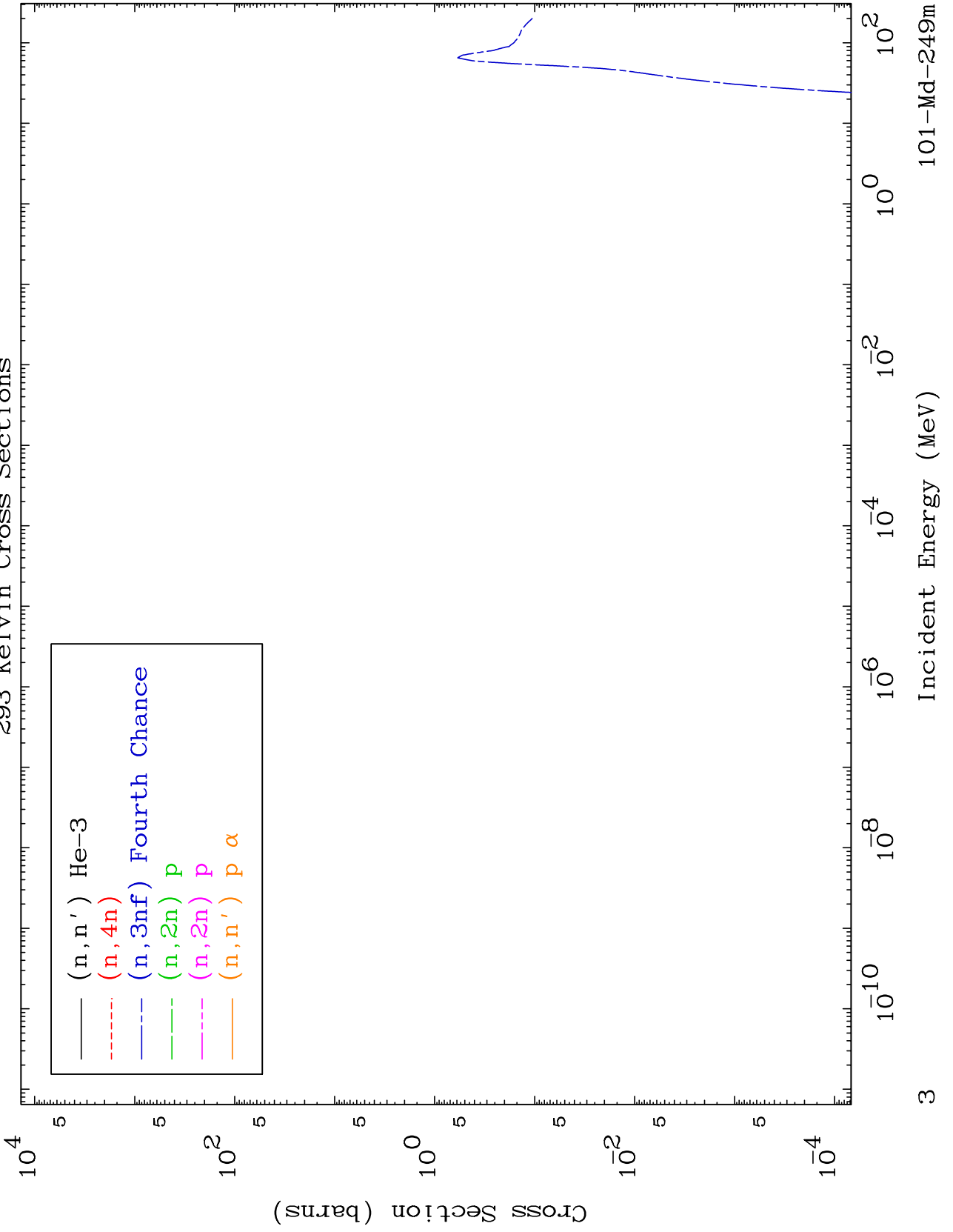
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MAT 9955

Neutron Absorption
293 Kelvin Cross Sections

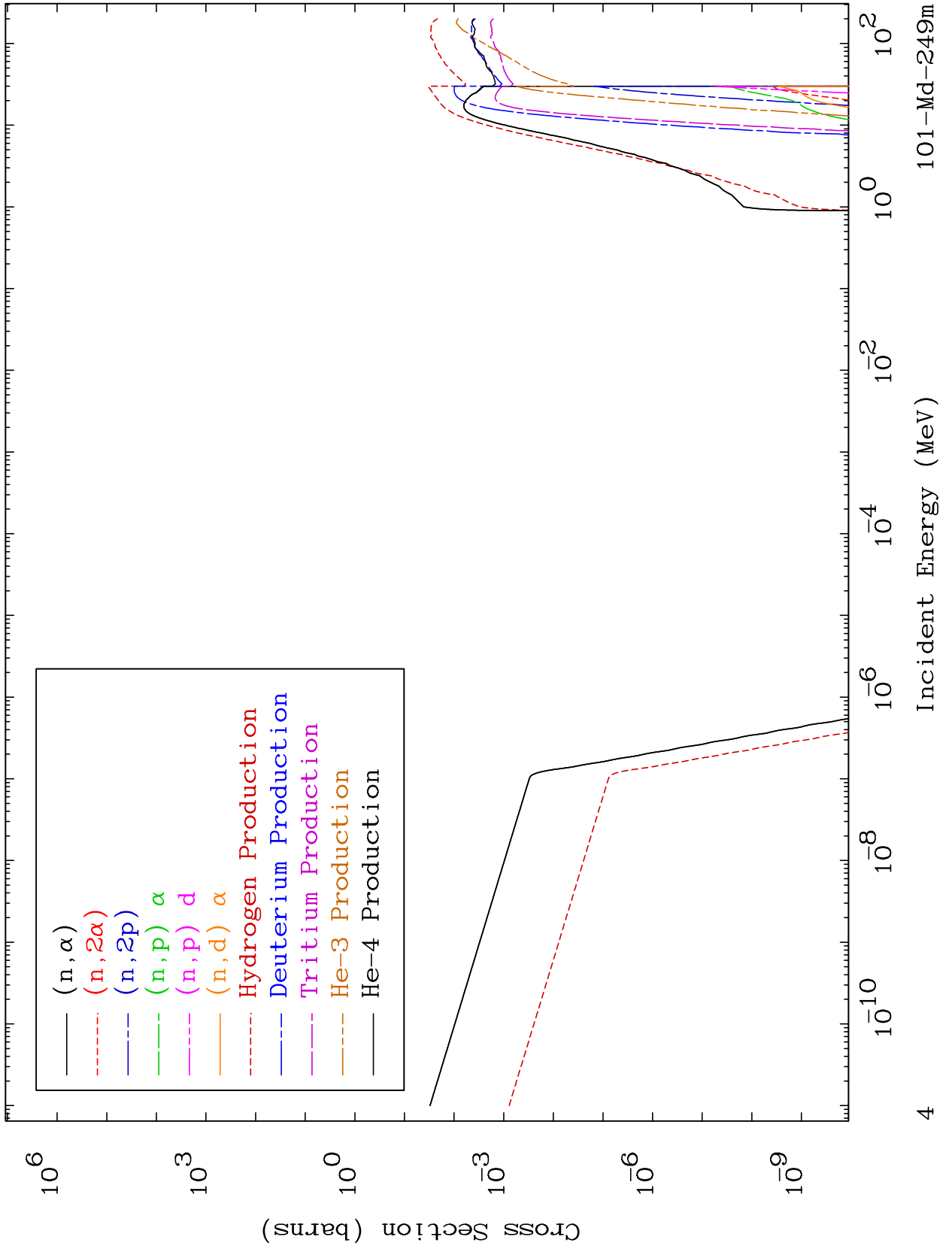
101-Md-249m



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Neutron Absorption
293 Kelvin Cross Sections

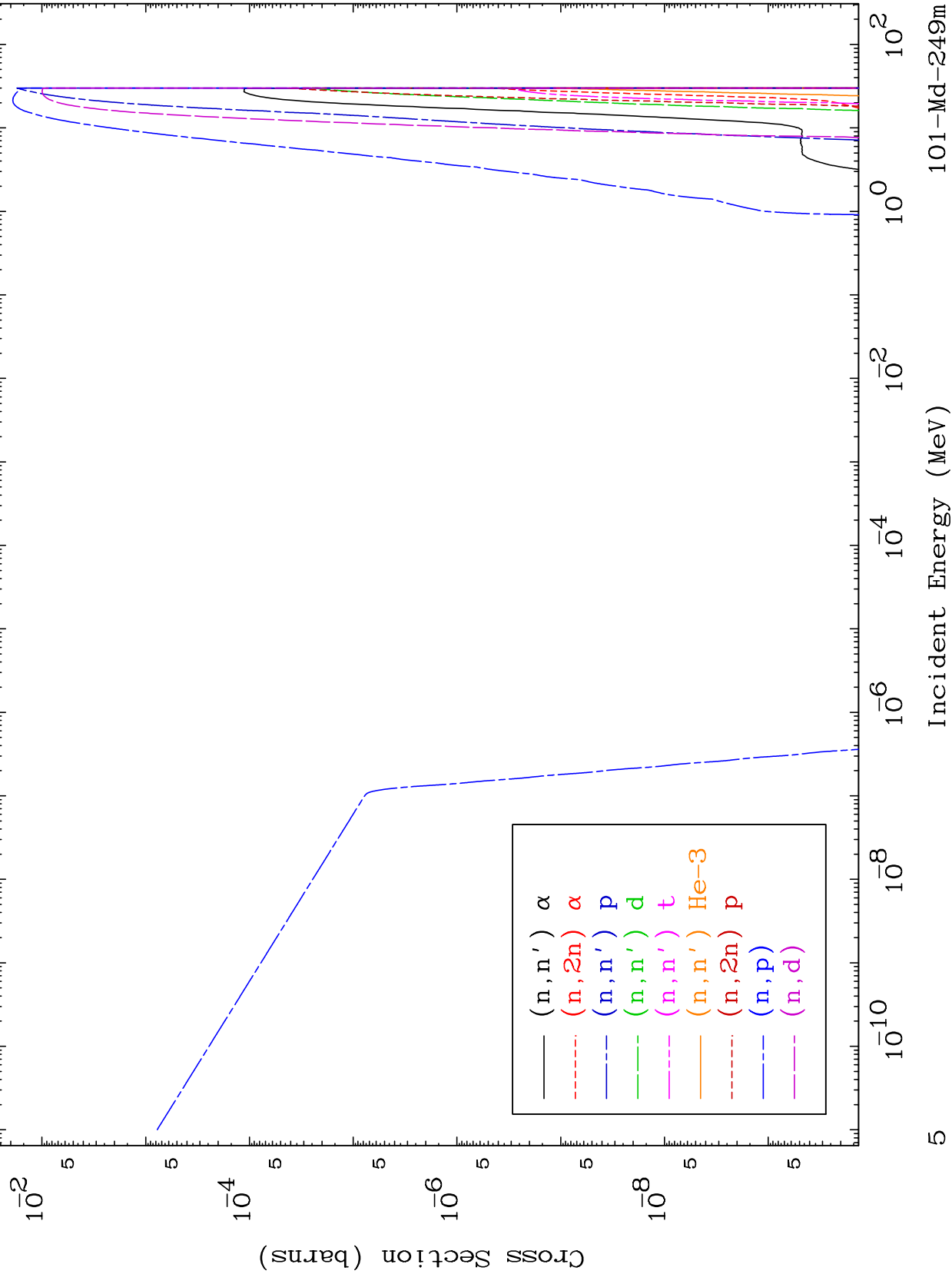
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MAT 9955

Charged Particle
293 Kelvin Cross Sections

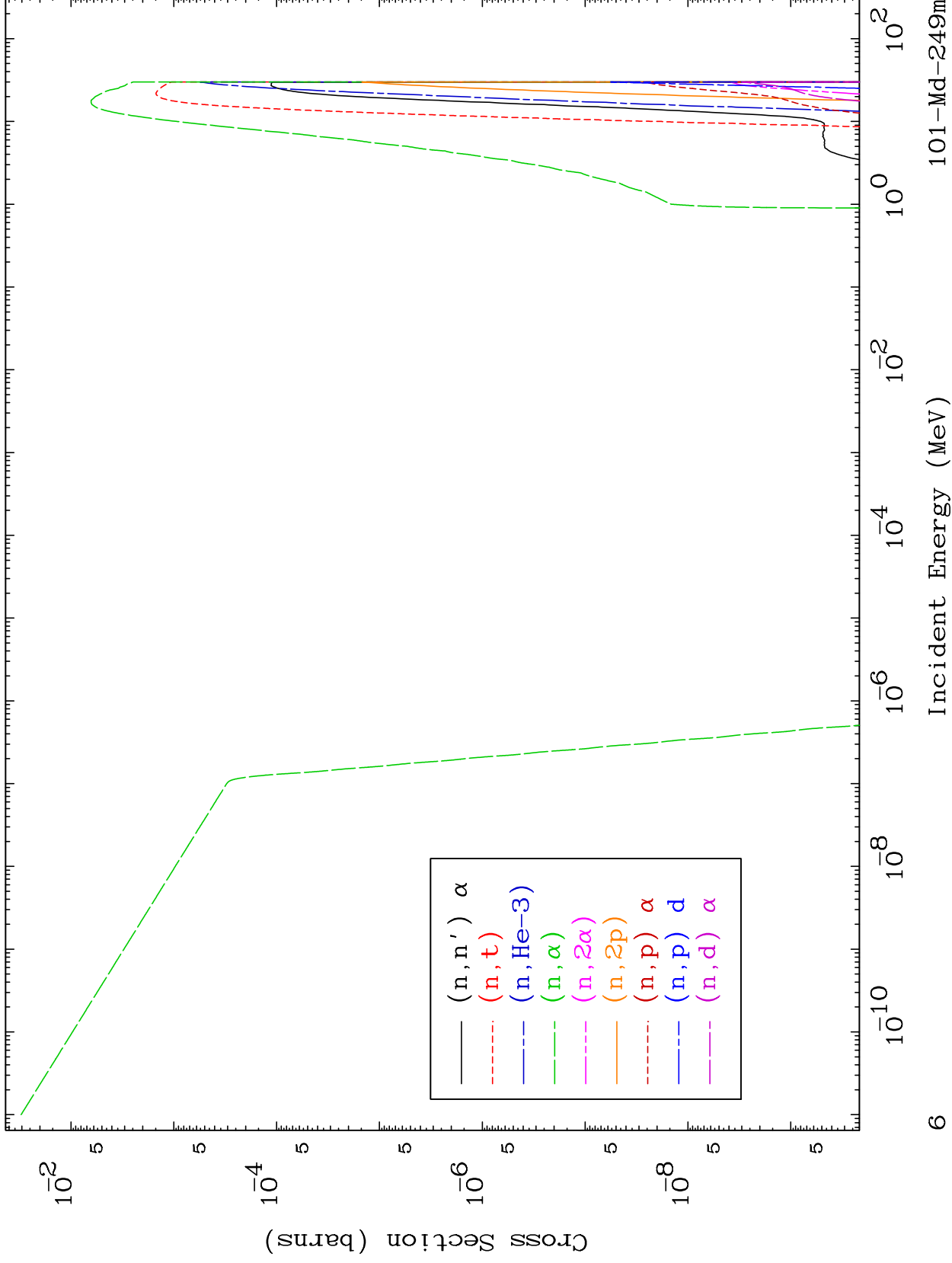
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Charged Particle
293 Kelvin Cross Sections

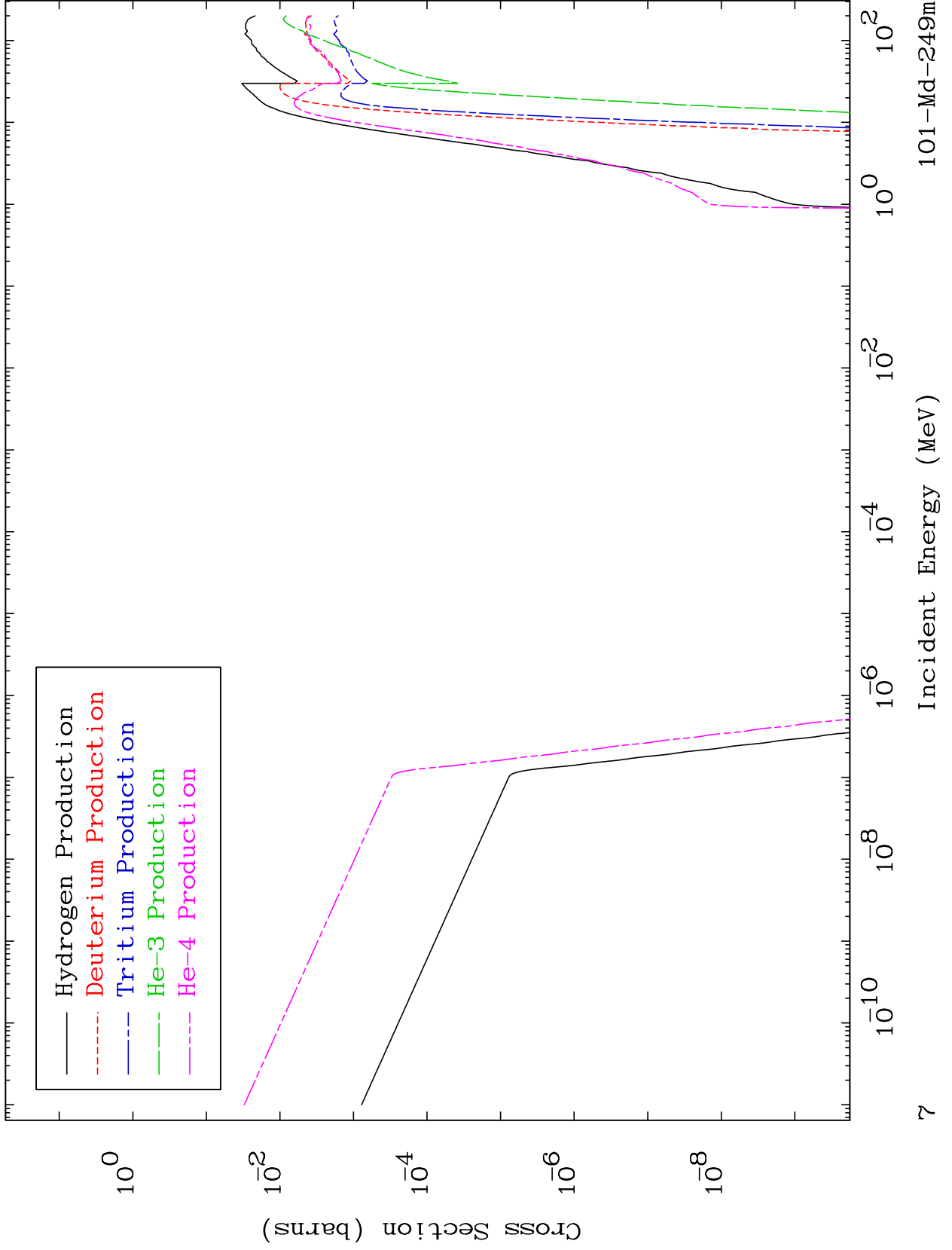
101-Md-249m



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Particle Production
293 Kelvin Cross Sections

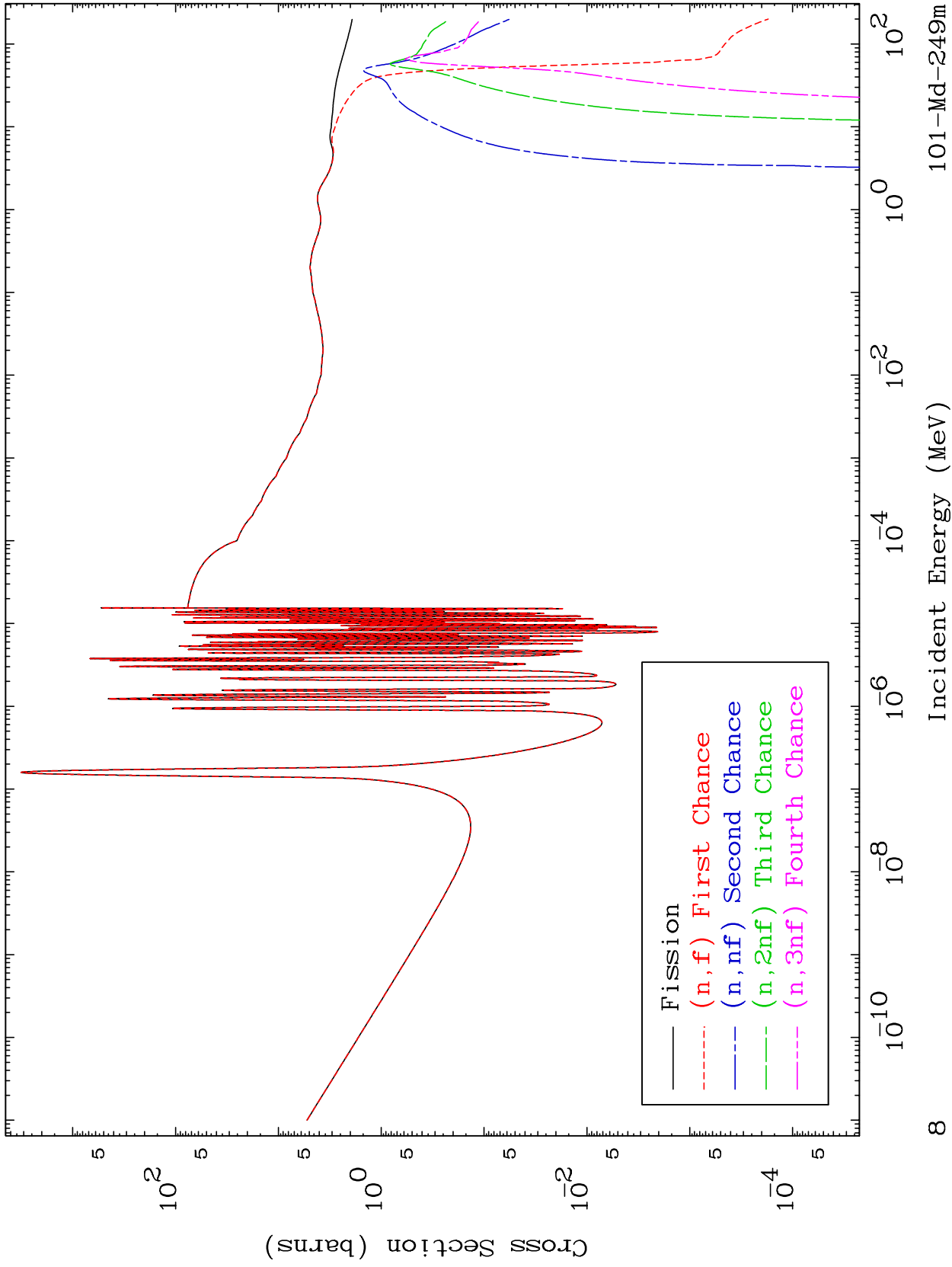
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Fission
293 Kelvin Cross Sections

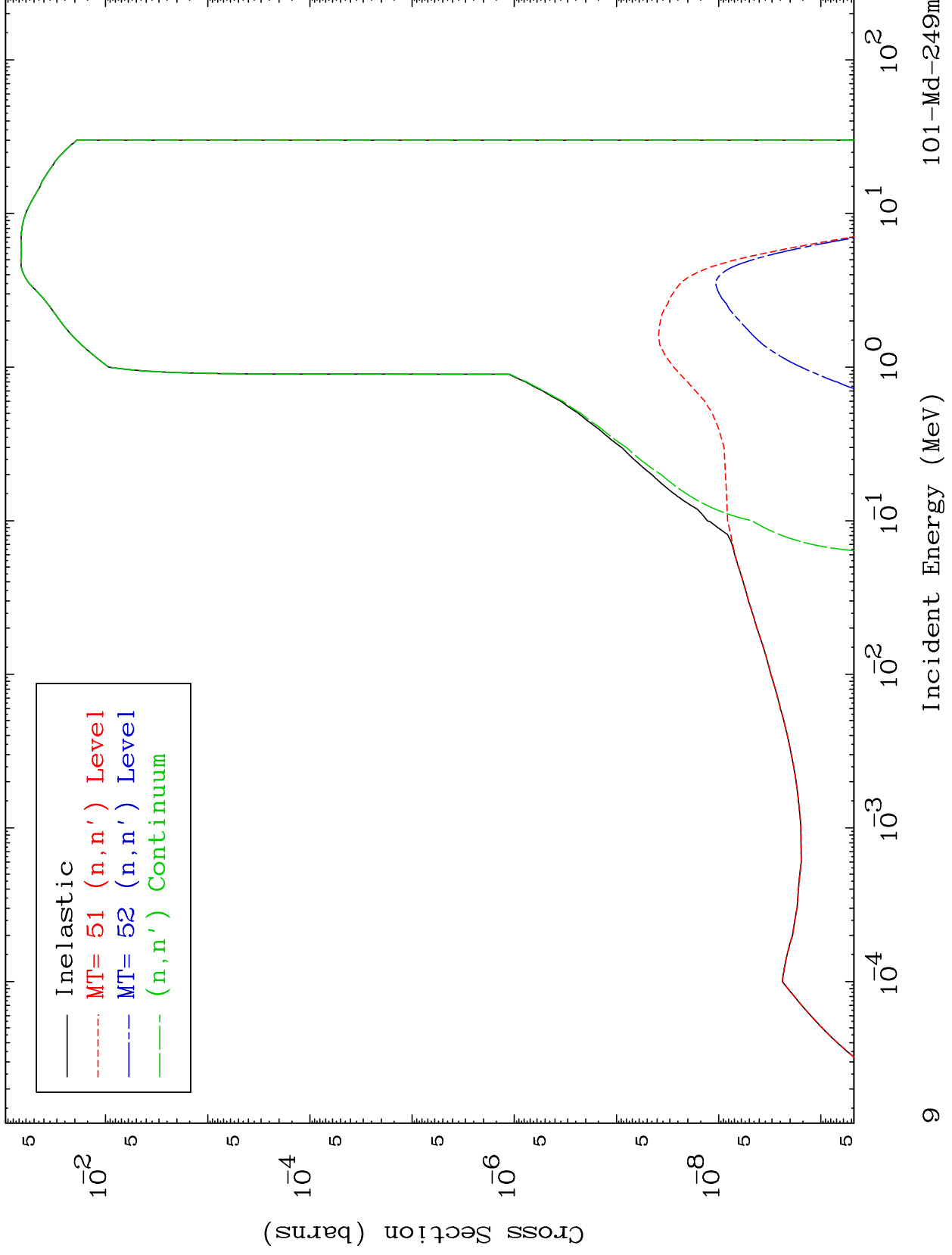
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MAT 9955

(n,n') Levels
293 Kelvin Cross Sections

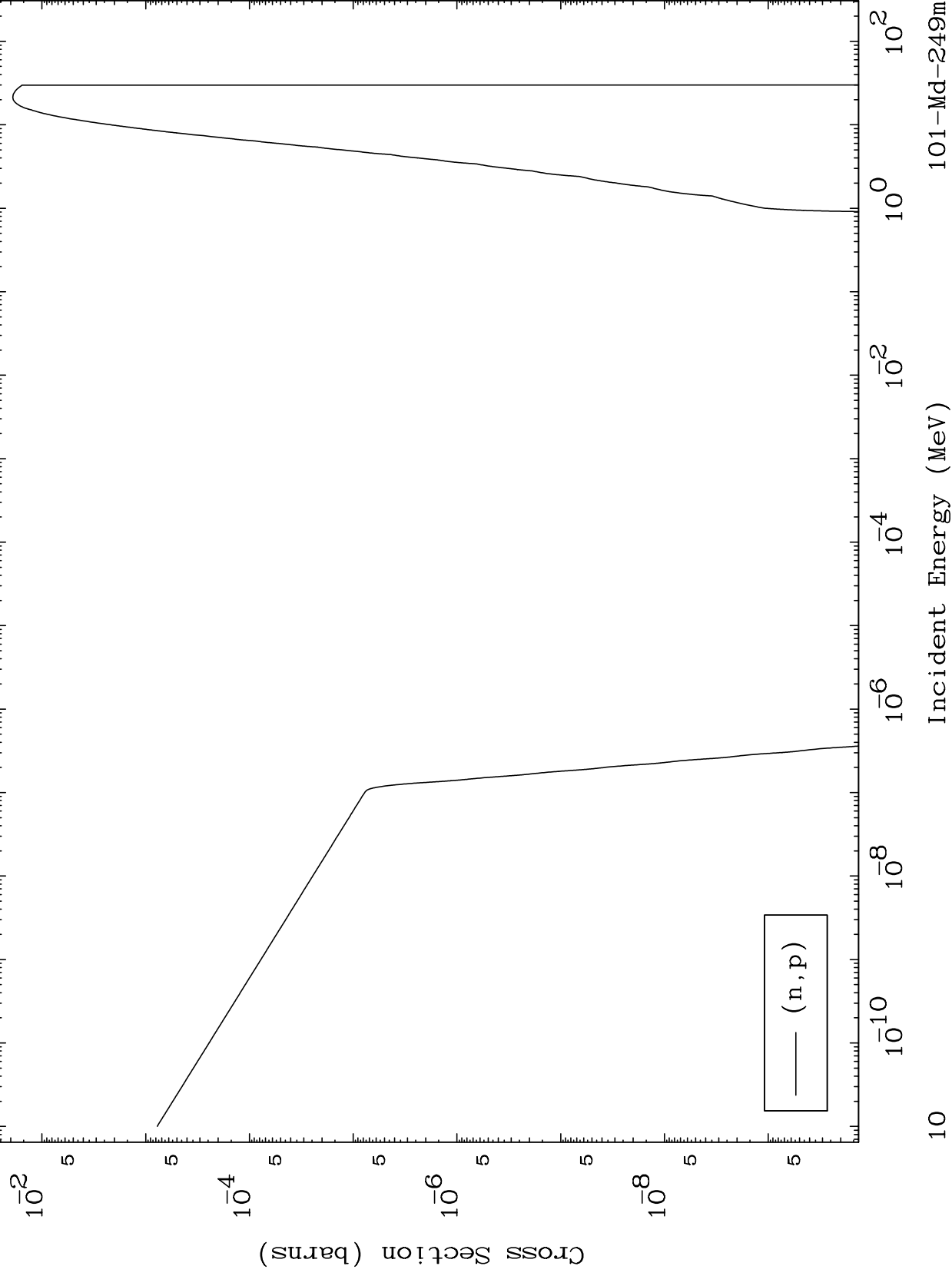
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(n,p) Levels
293 Kelvin Cross Sections

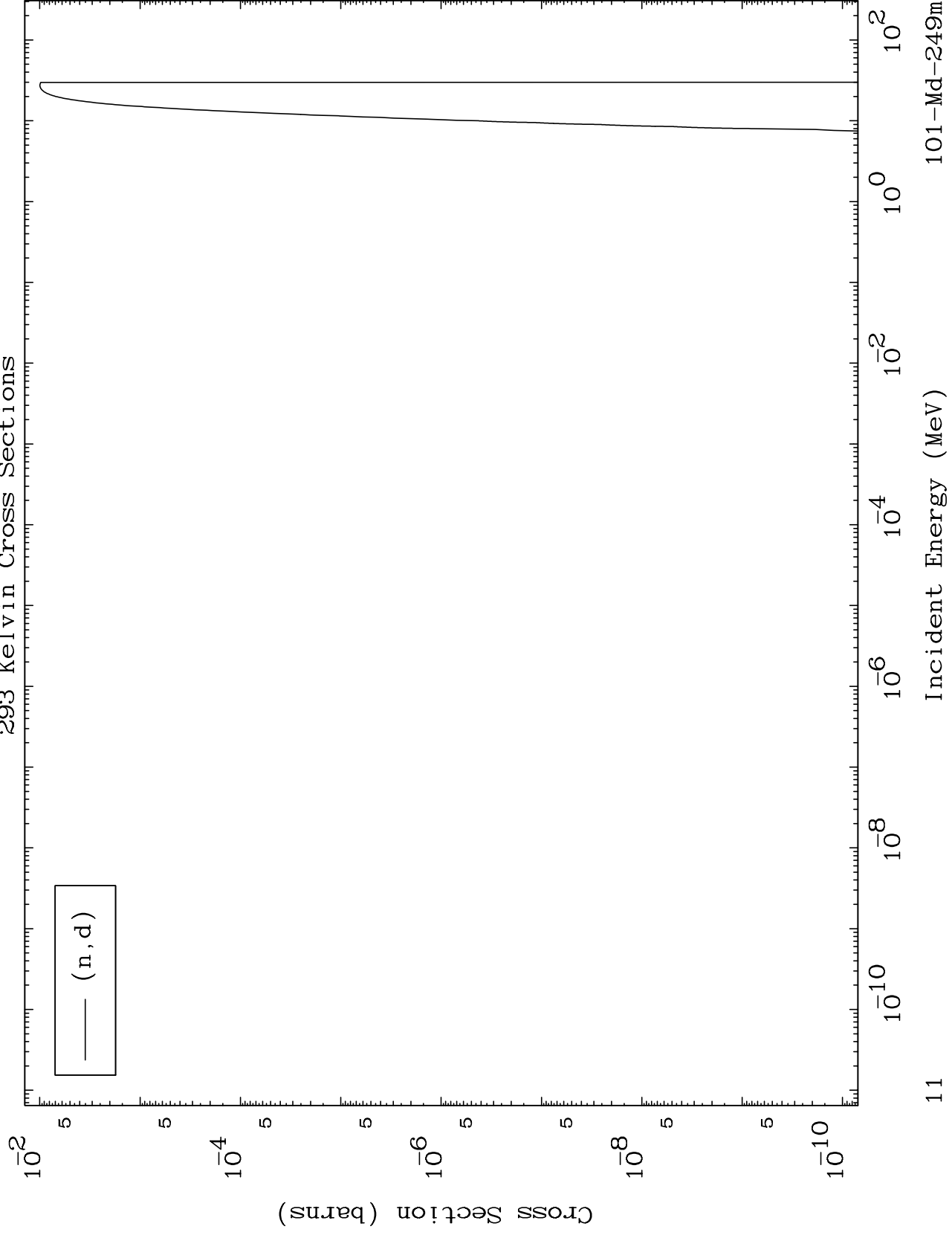
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(n,d) Levels
293 Kelvin Cross Sections

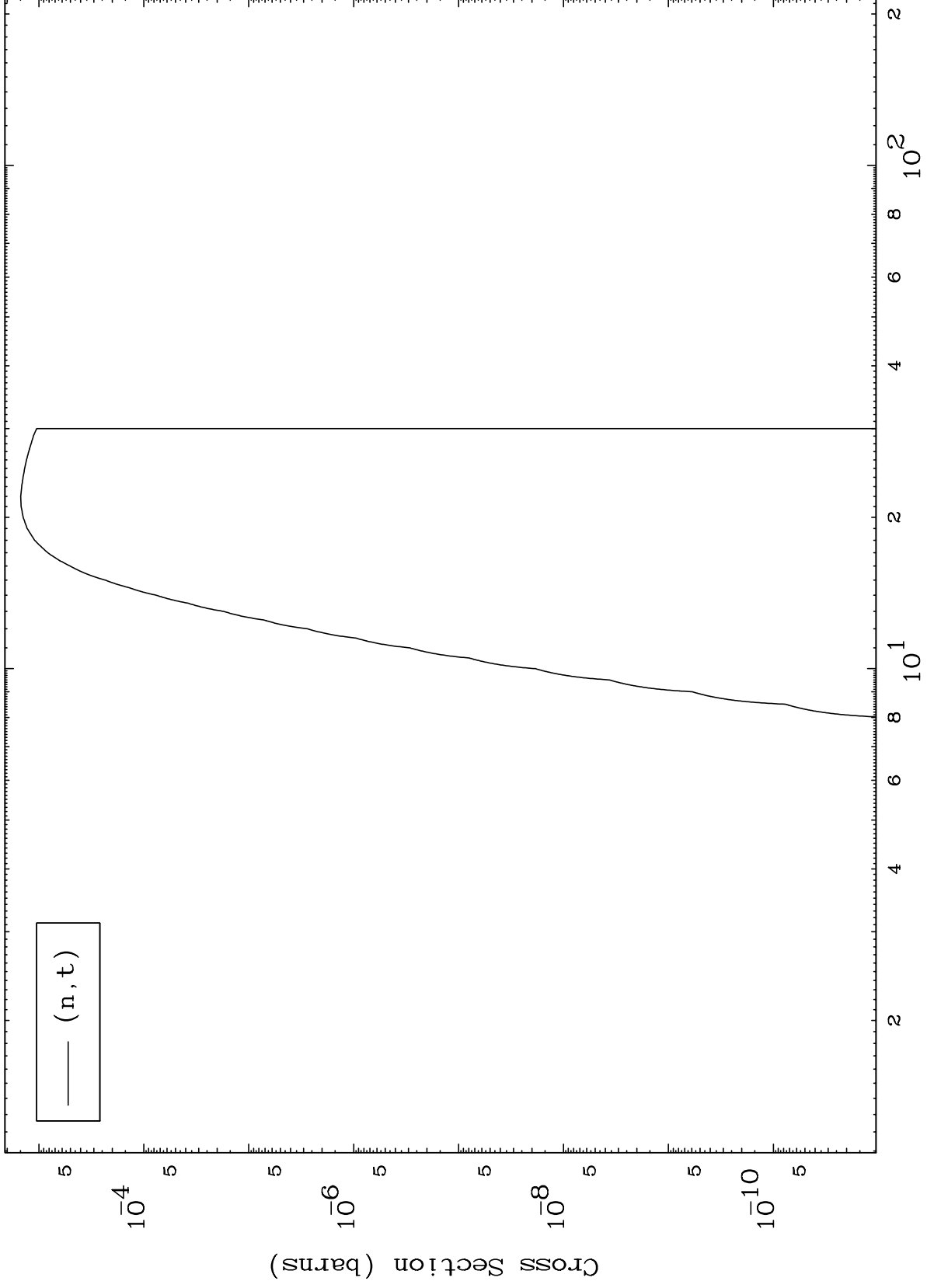
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(n,t) Levels
293 Kelvin Cross Sections

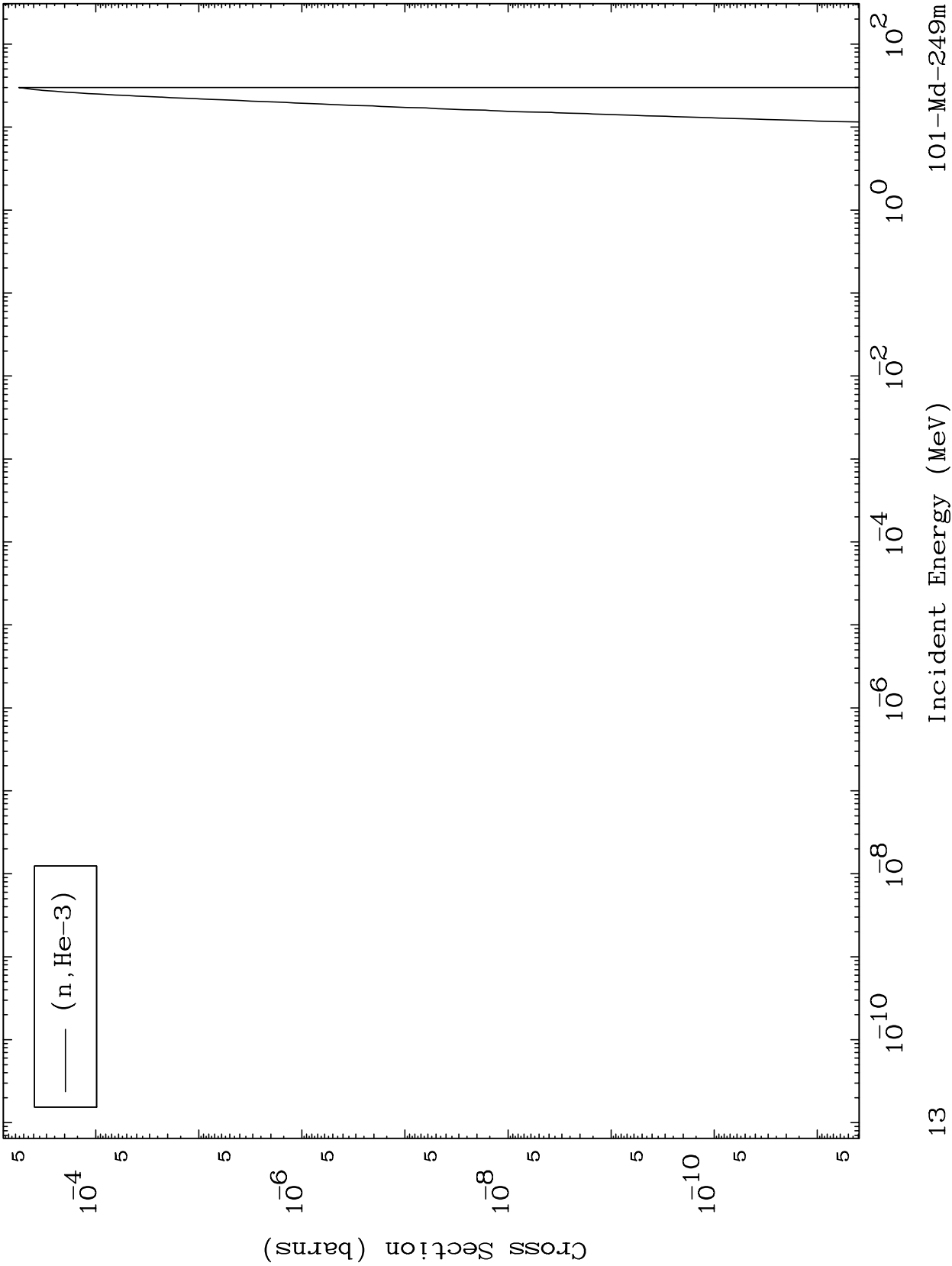
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(n,He3) Levels
293 Kelvin Cross Sections

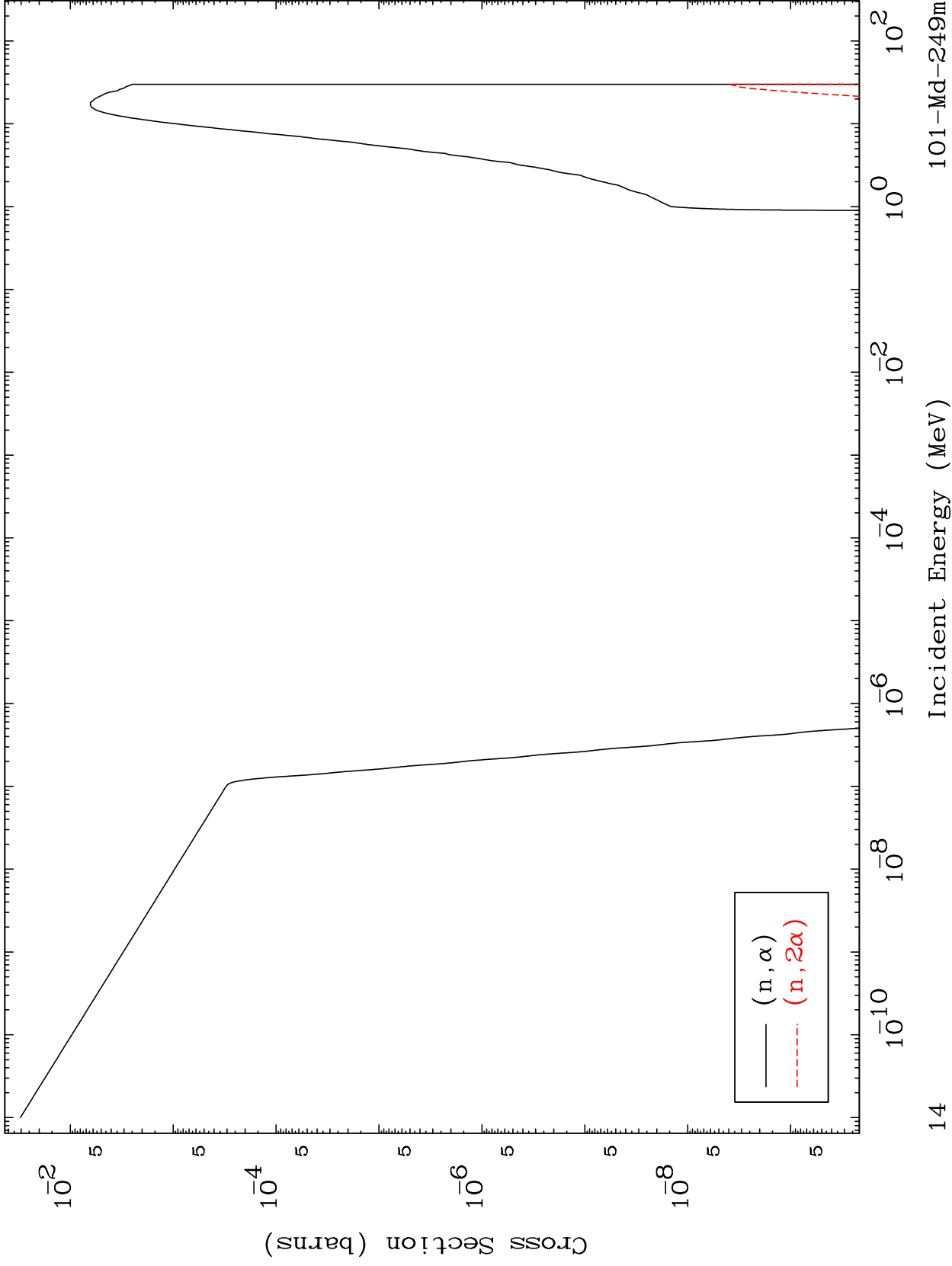
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MAT 9955

(n, α) Levels
293 Kelvin Cross Sections

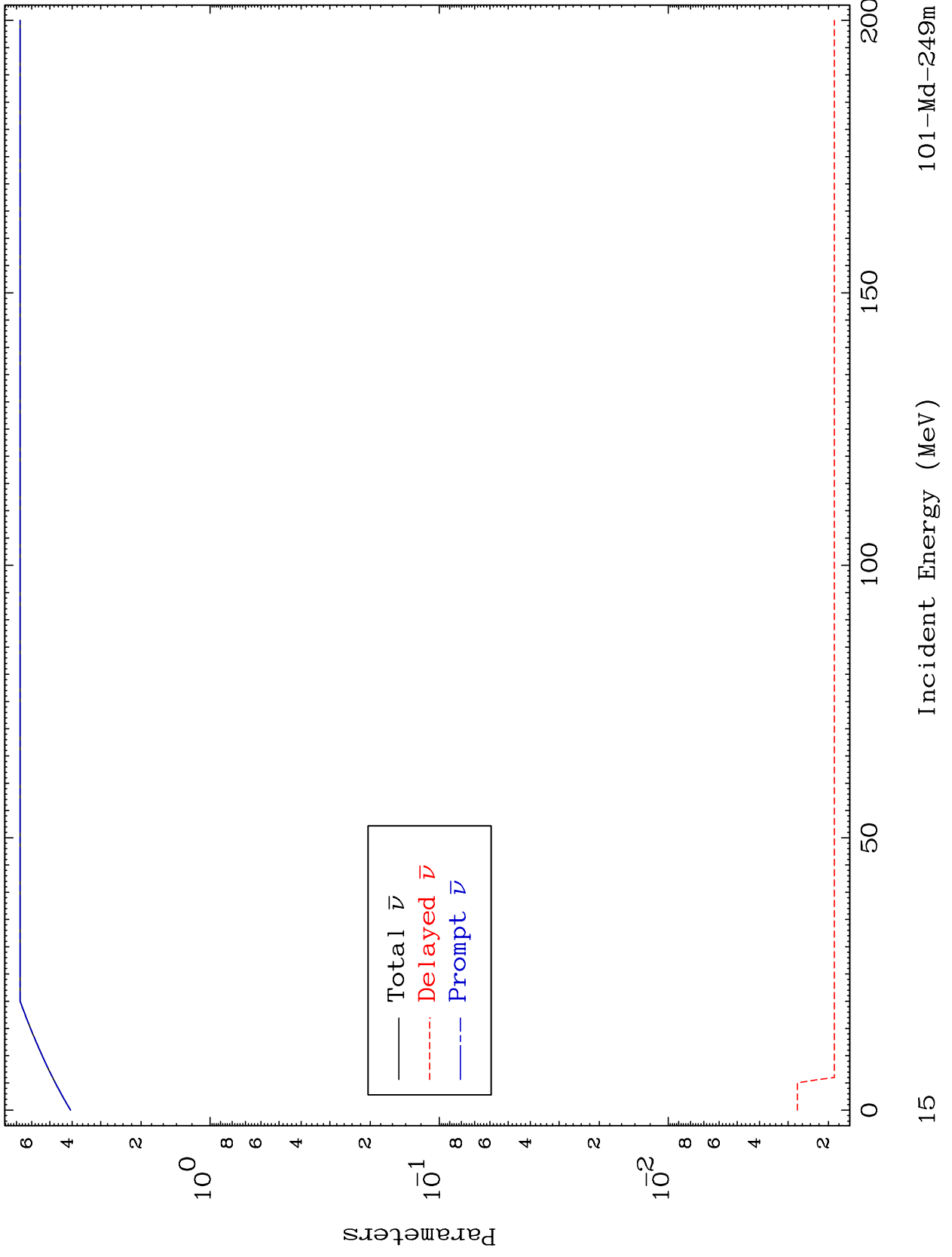
101-Md-249m



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Energy Release
Parameters

101-Md-249m



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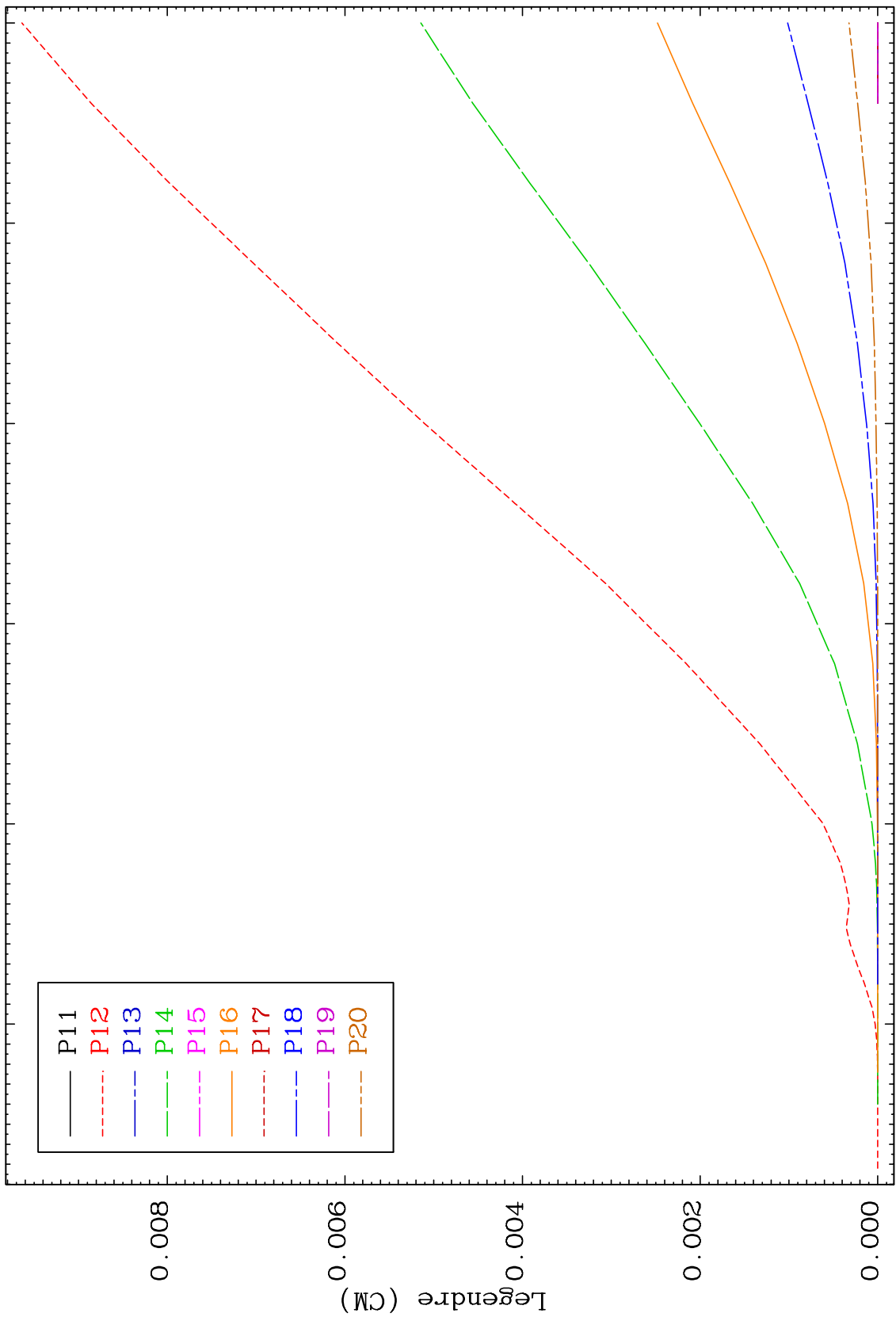
Incident Energy (MeV)

101-Md-249m

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Elastic Legendre Coefficients

101-Md-249m



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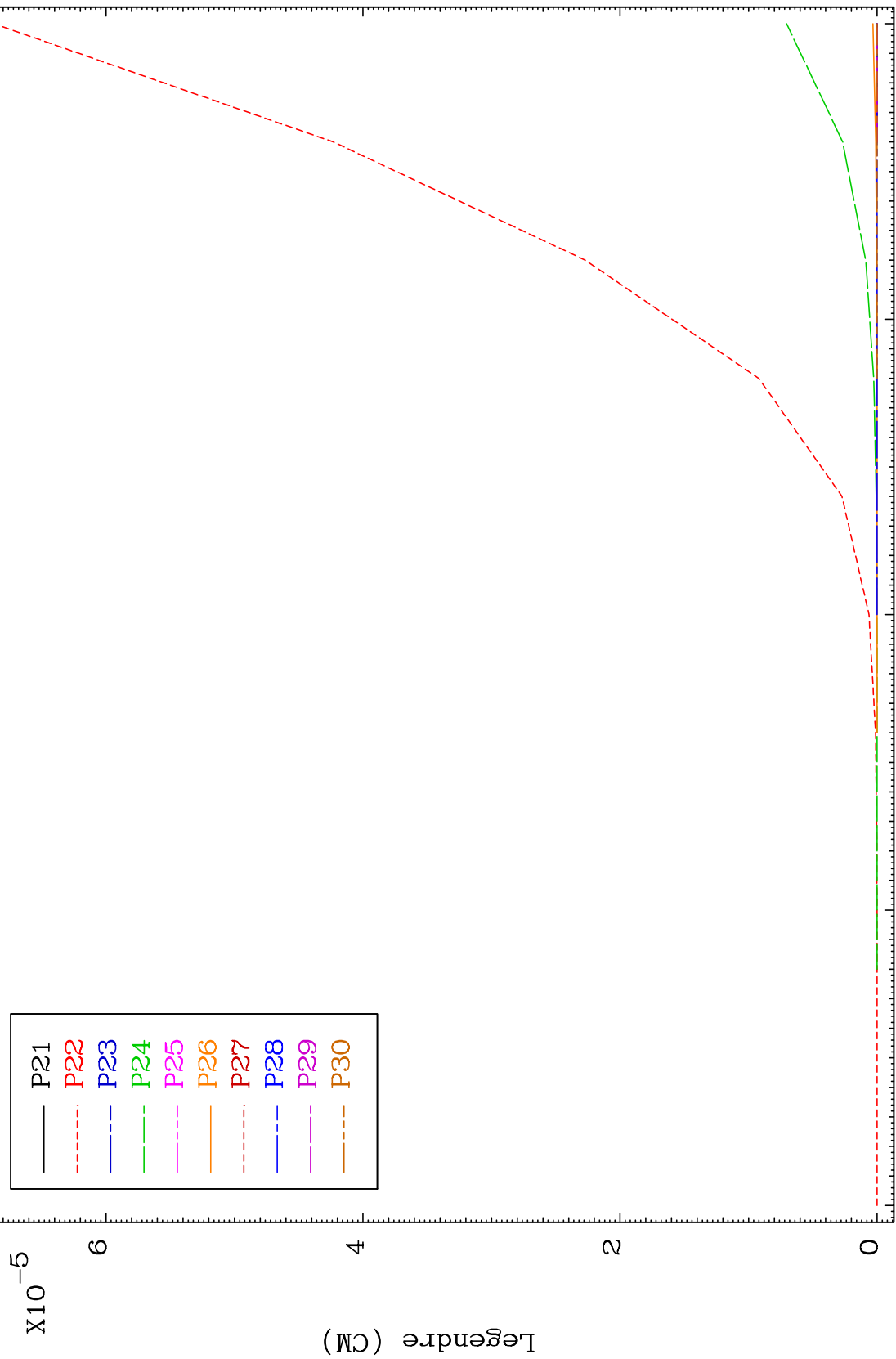
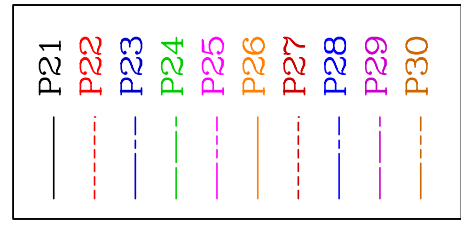
101-Md-249m

Incident Energy (MeV)

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Elastic
Legendre Coefficients

101-Md-249m



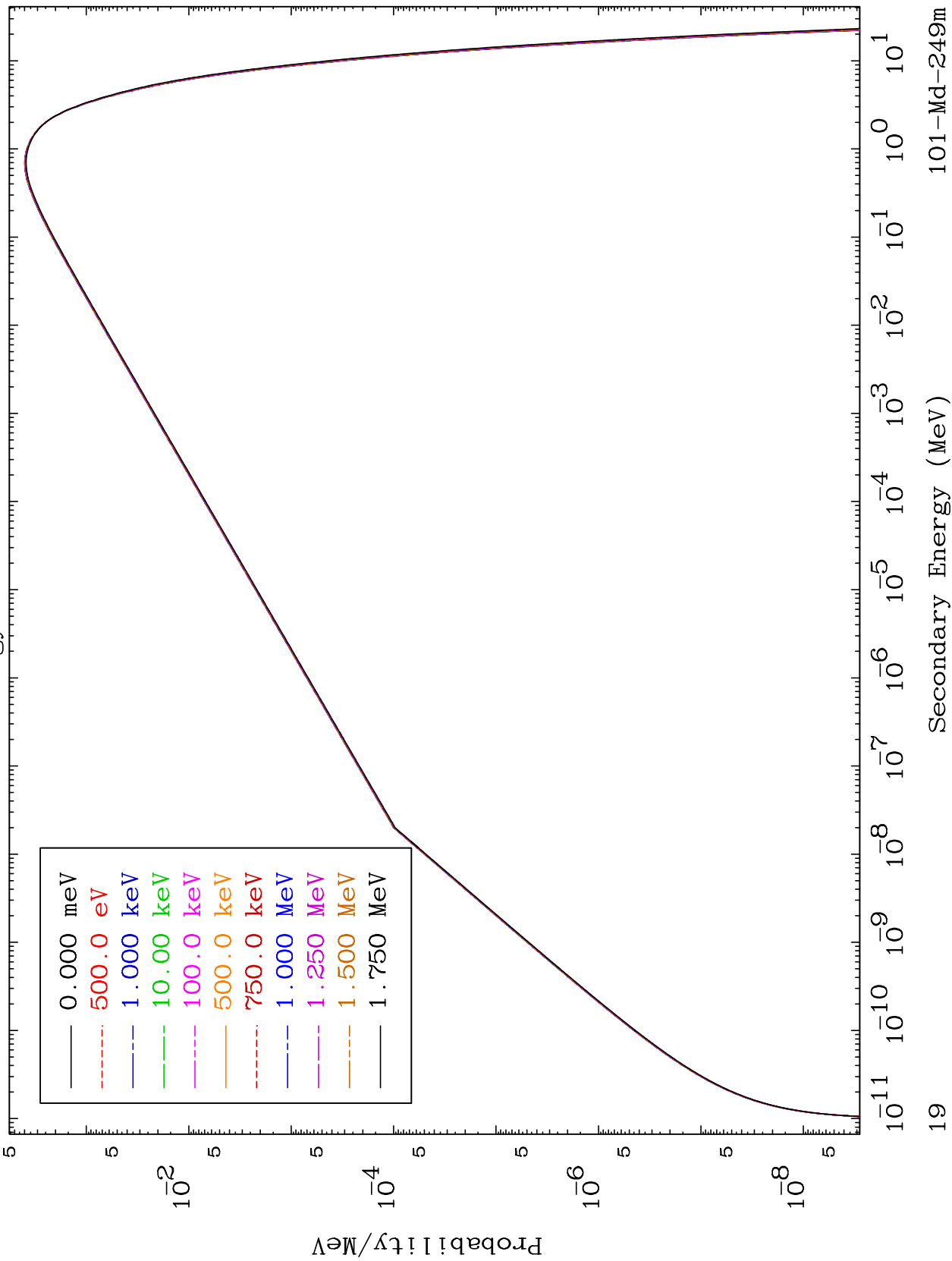
Incident Energy (MeV)

101-Md-249m

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Fission Energy Distributions

101-Md-249m



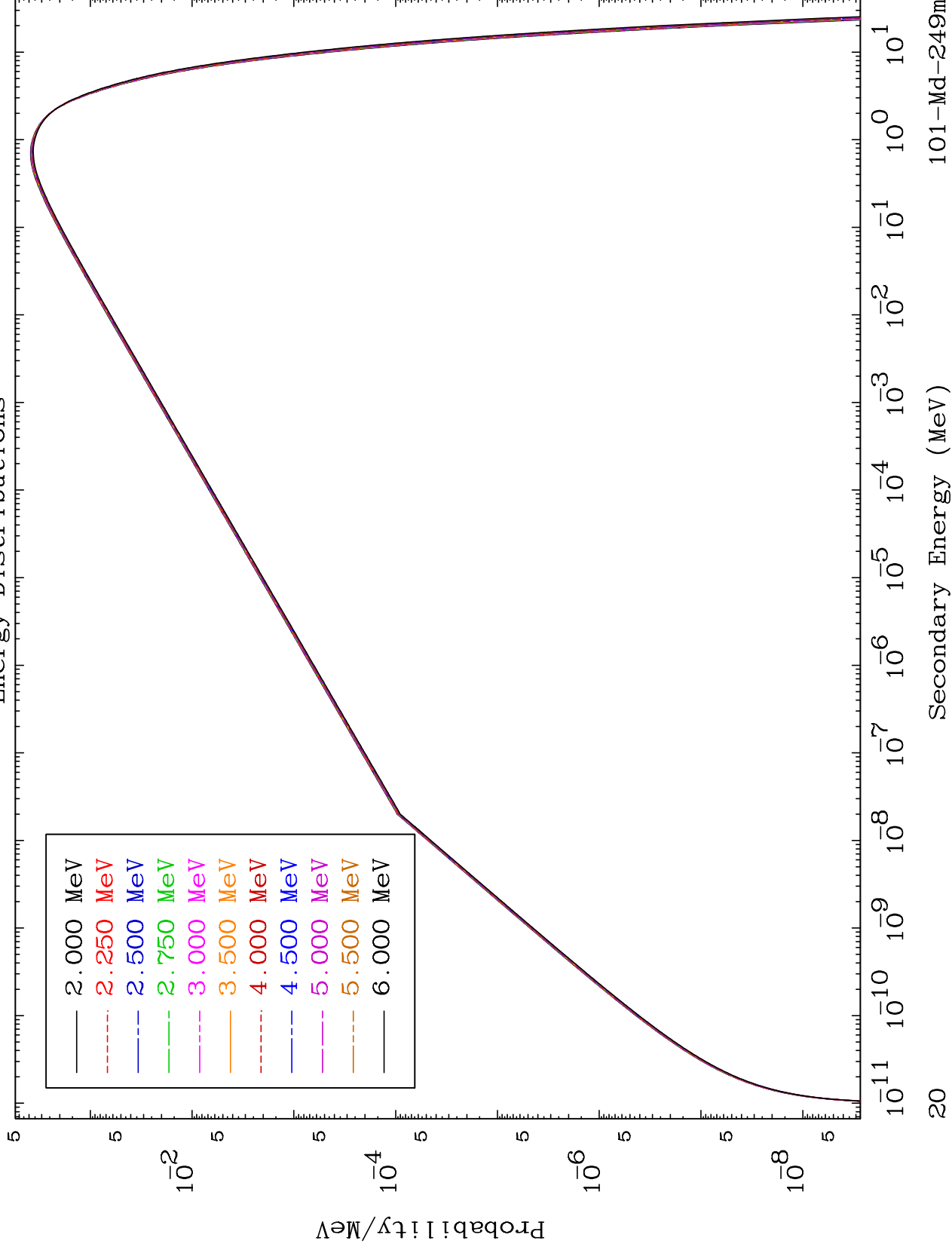
101-Md-249m

Secondary Energy (MeV)

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Fission Energy Distributions

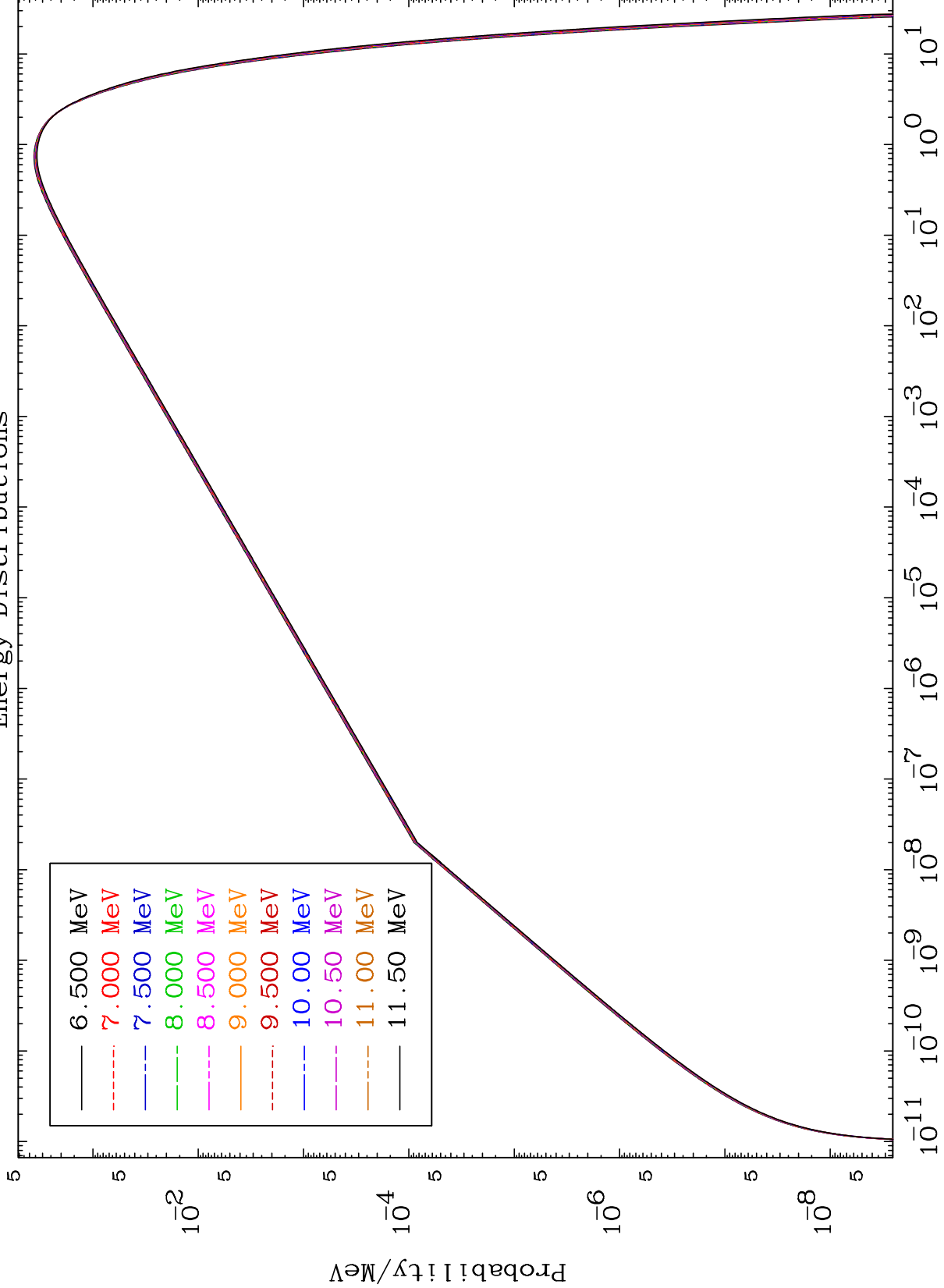
101-Md-249m



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Fission Energy Distributions

101-Md-249m



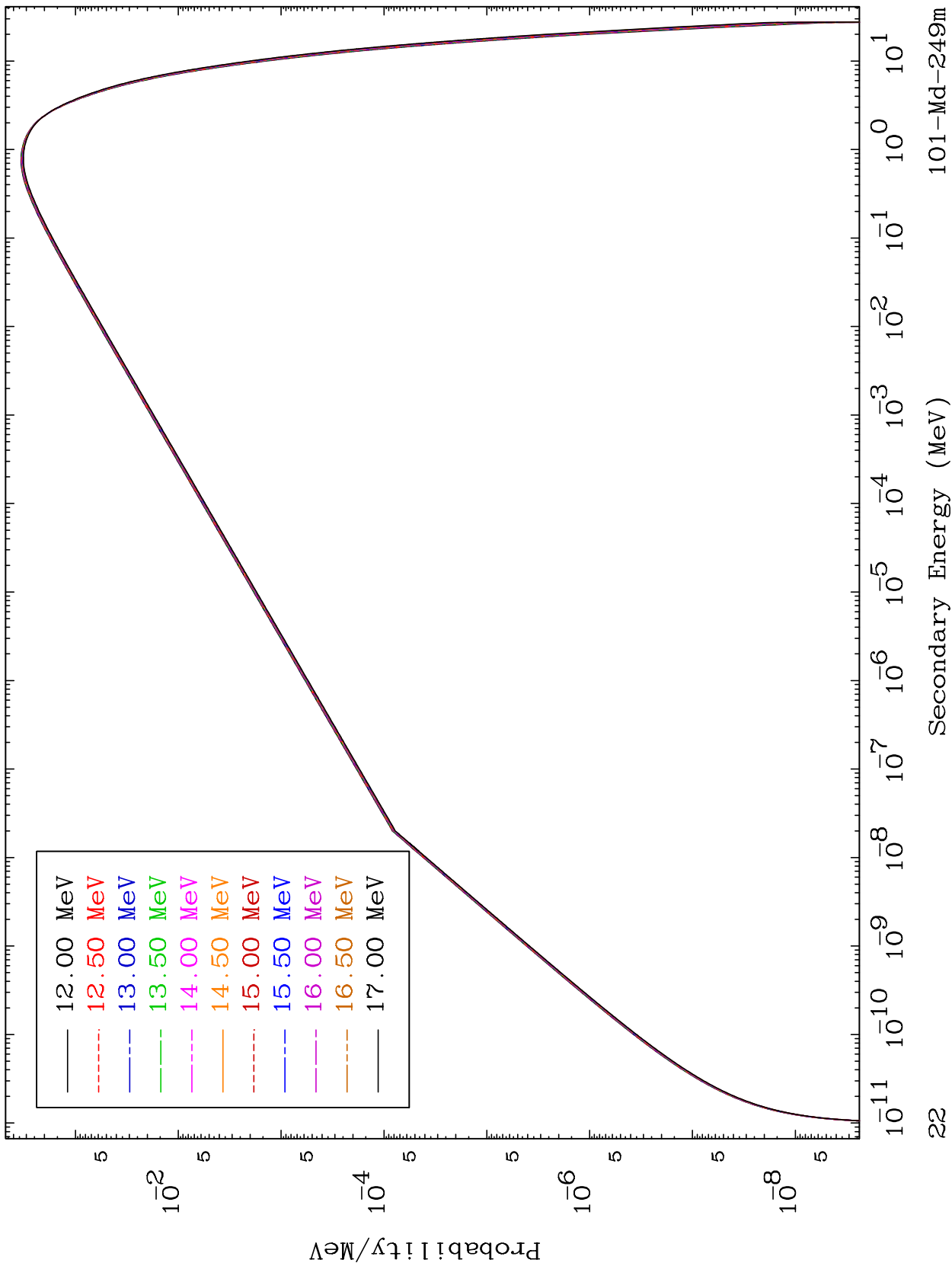
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Secondary Energy (MeV)

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Fission Energy Distributions

101-Md-249m



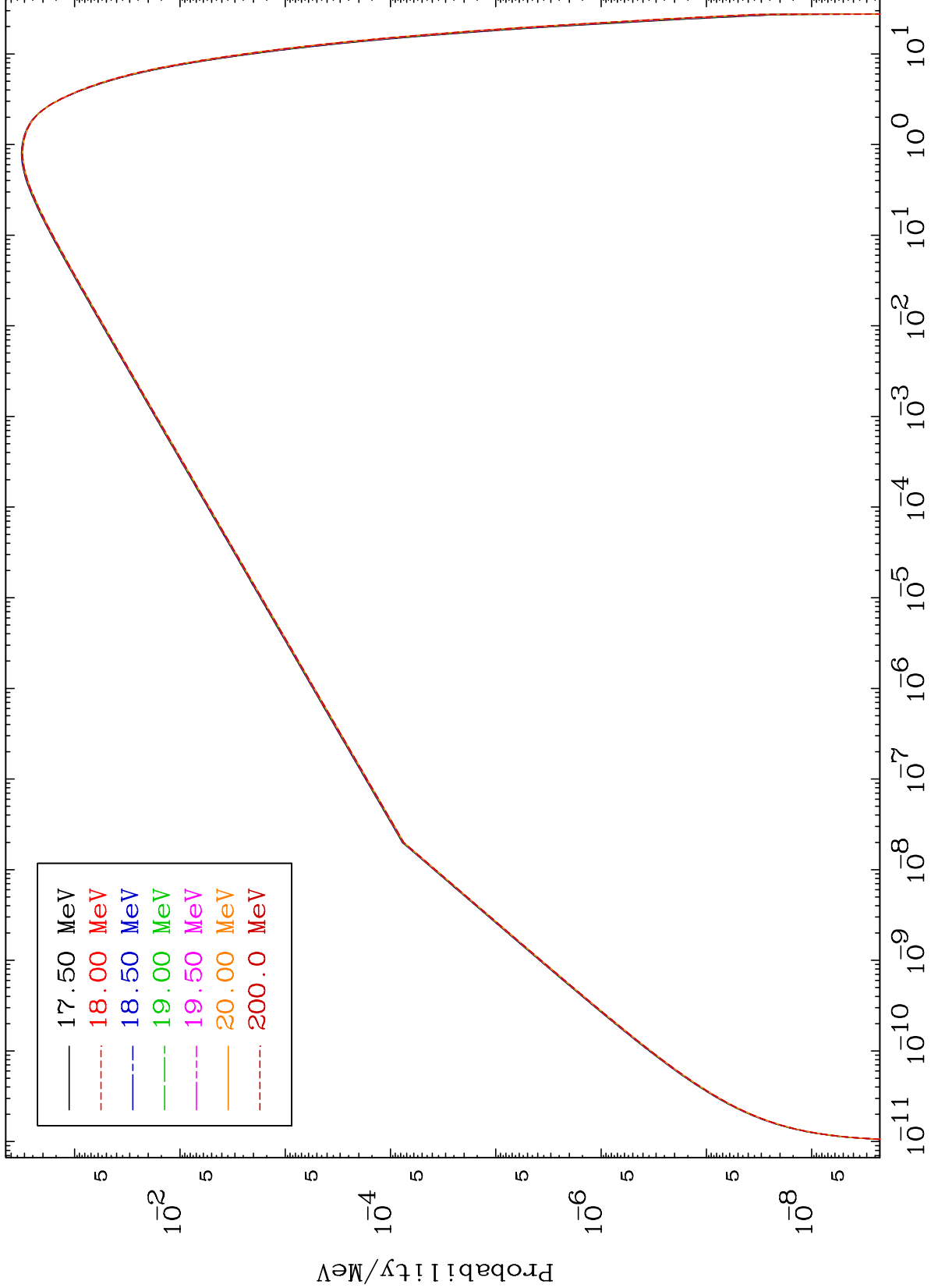
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Secondary Energy (MeV)

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Fission Energy Distributions

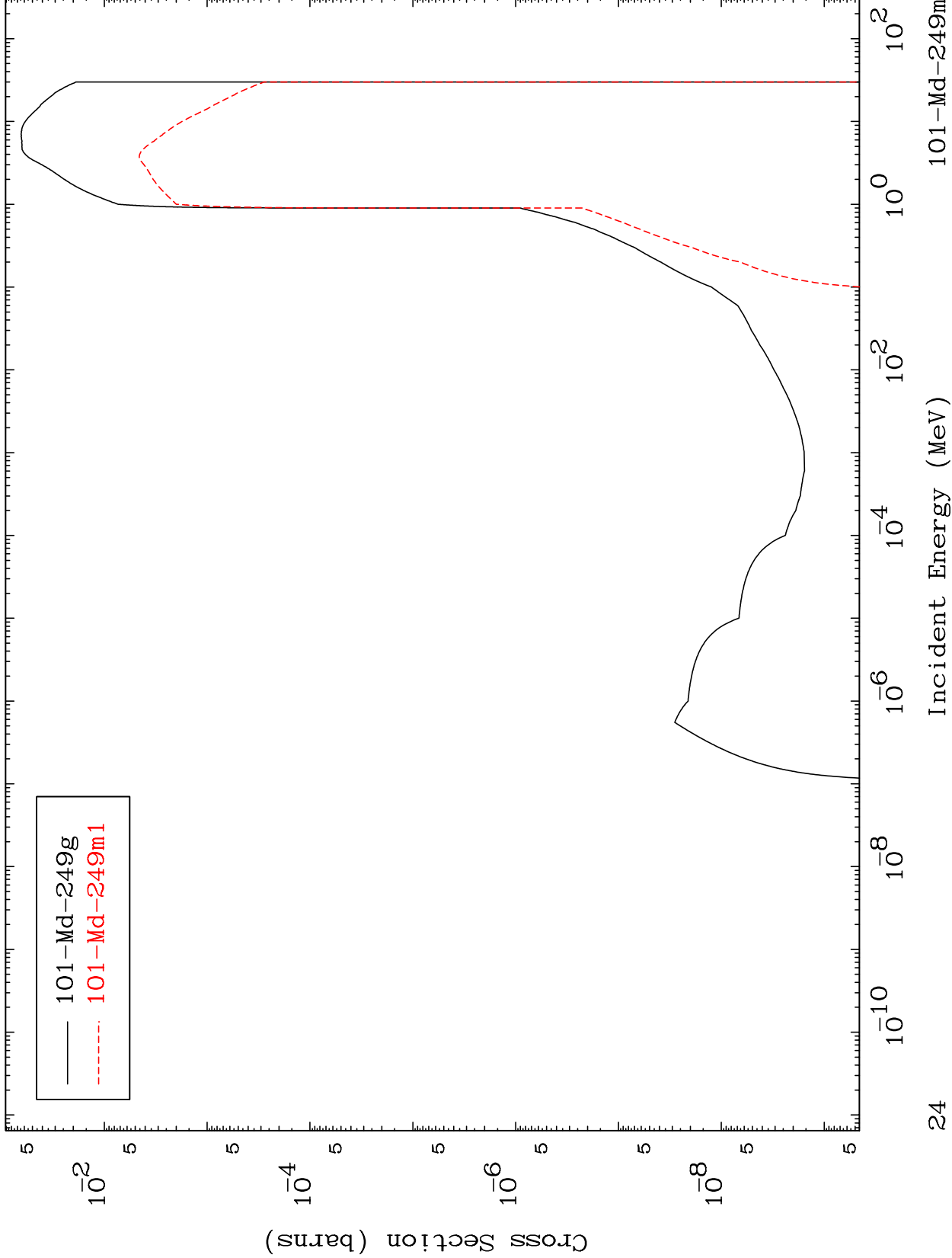
101-Md-249m



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Inelastic
Radionuclide Production Cross Section

101-Md-249m

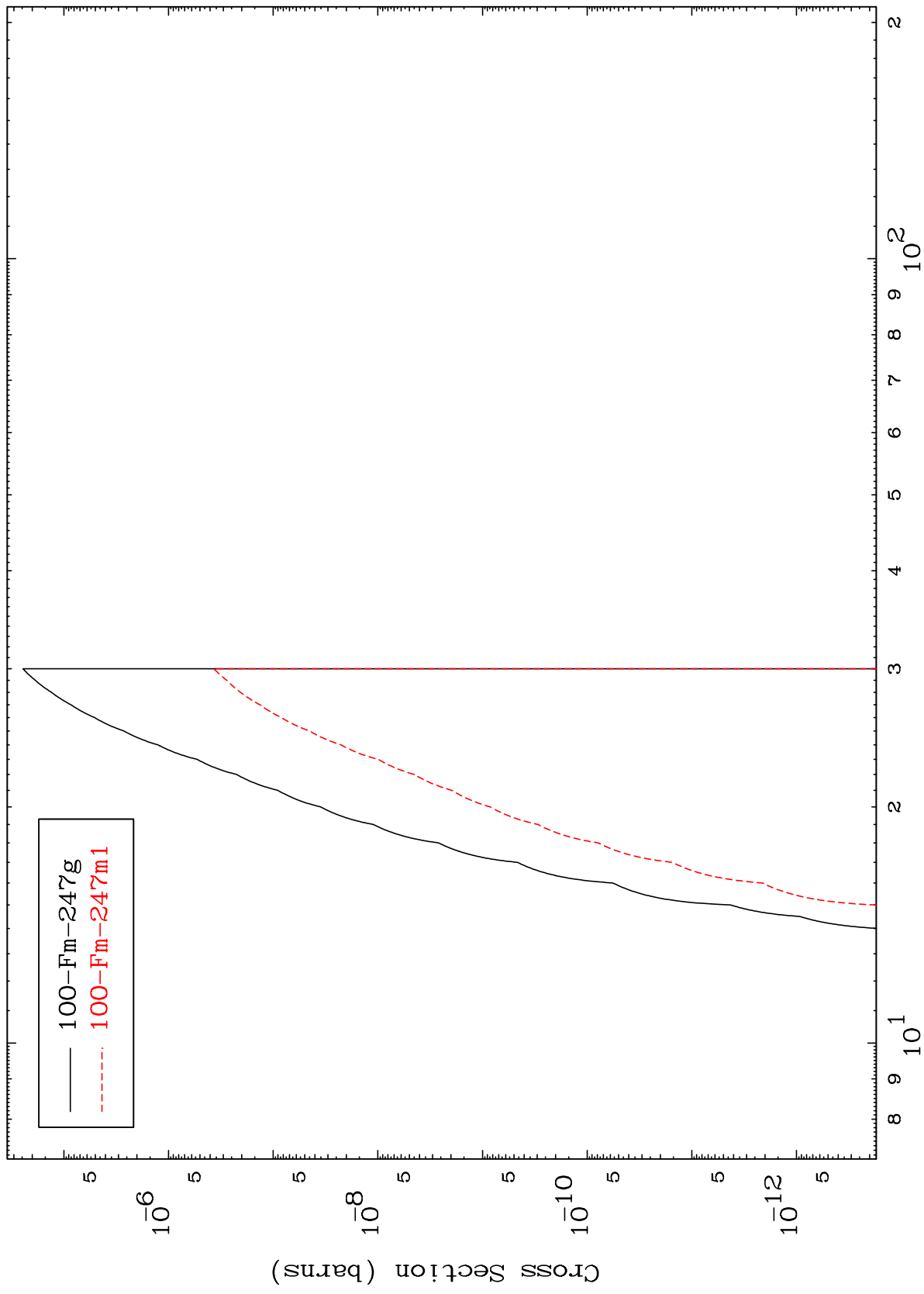


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101-Md-249m

(n,n') d

Radionuclide Production Cross Section



25

Incident Energy (MeV)

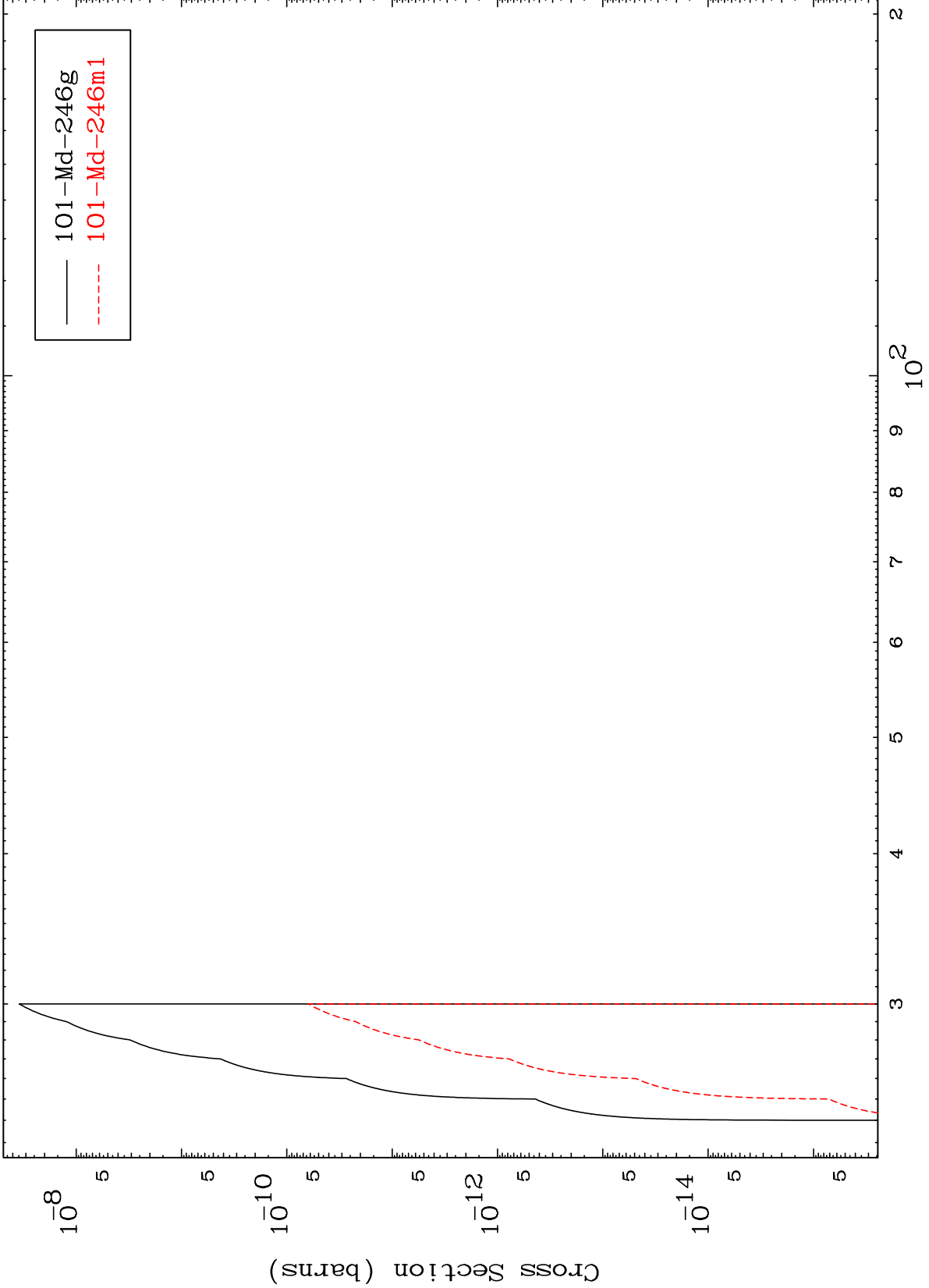
101-Md-249m

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(n,4n)

101-Md-249m

Radionuclide Production Cross Section



101-Md-246g
101-Md-246m1

26

Incident Energy (MeV)

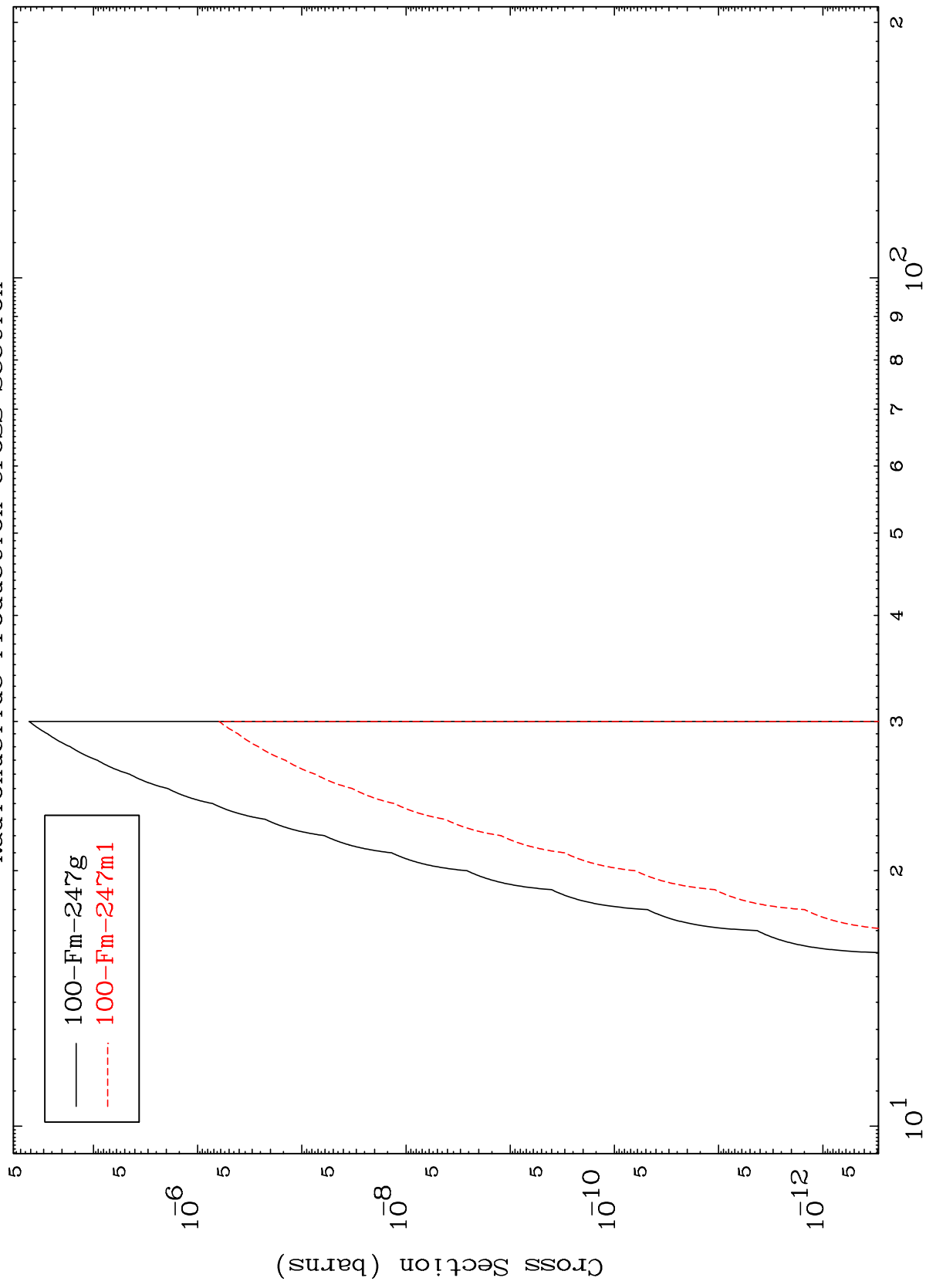
101-Md-249m

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(n,2n) p

101-Md-249m

Radionuclide Production Cross Section



Incident Energy (MeV)

101-Md-249m

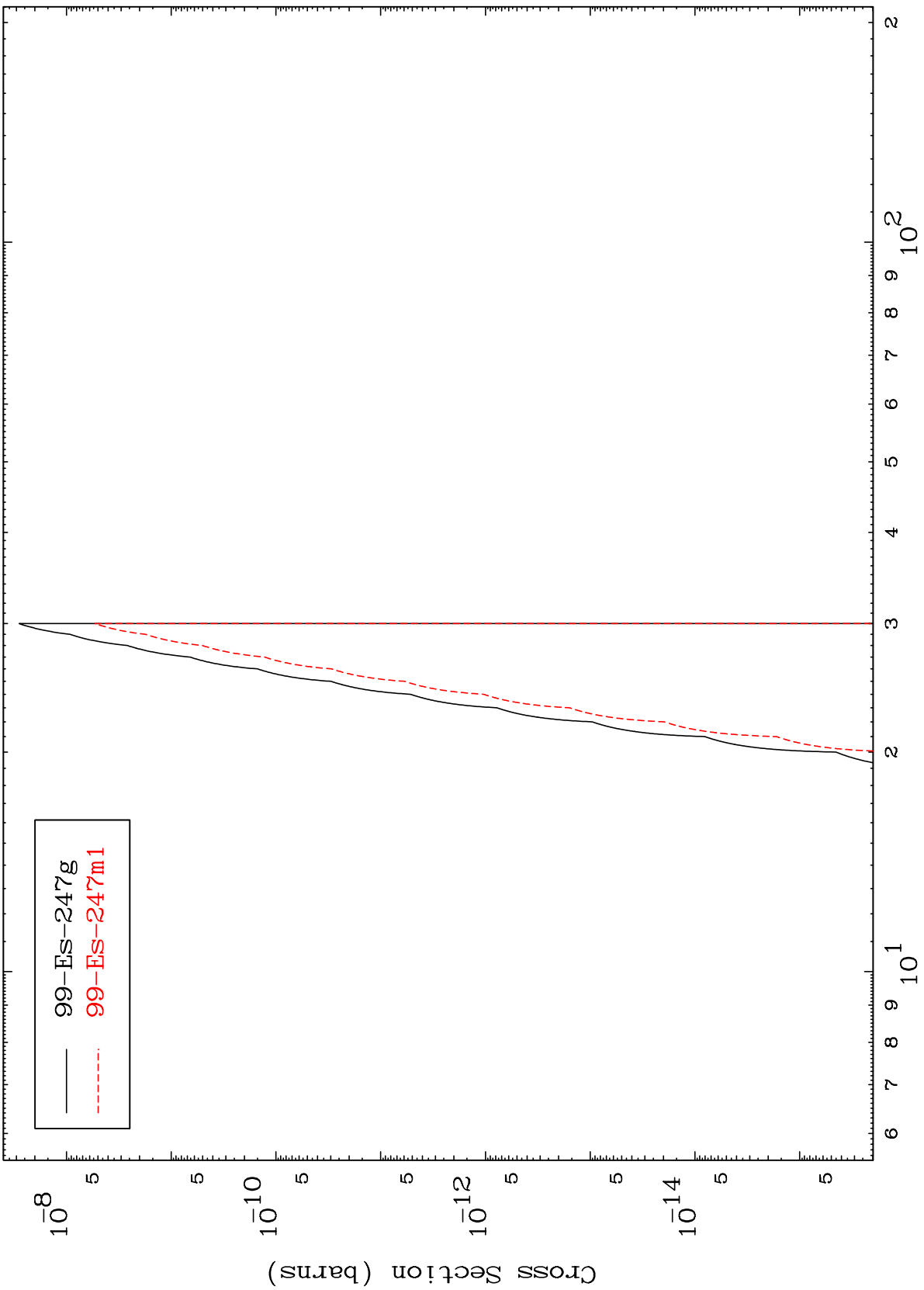
27

MAT 9955

(n,2n) p

101-Md-249m

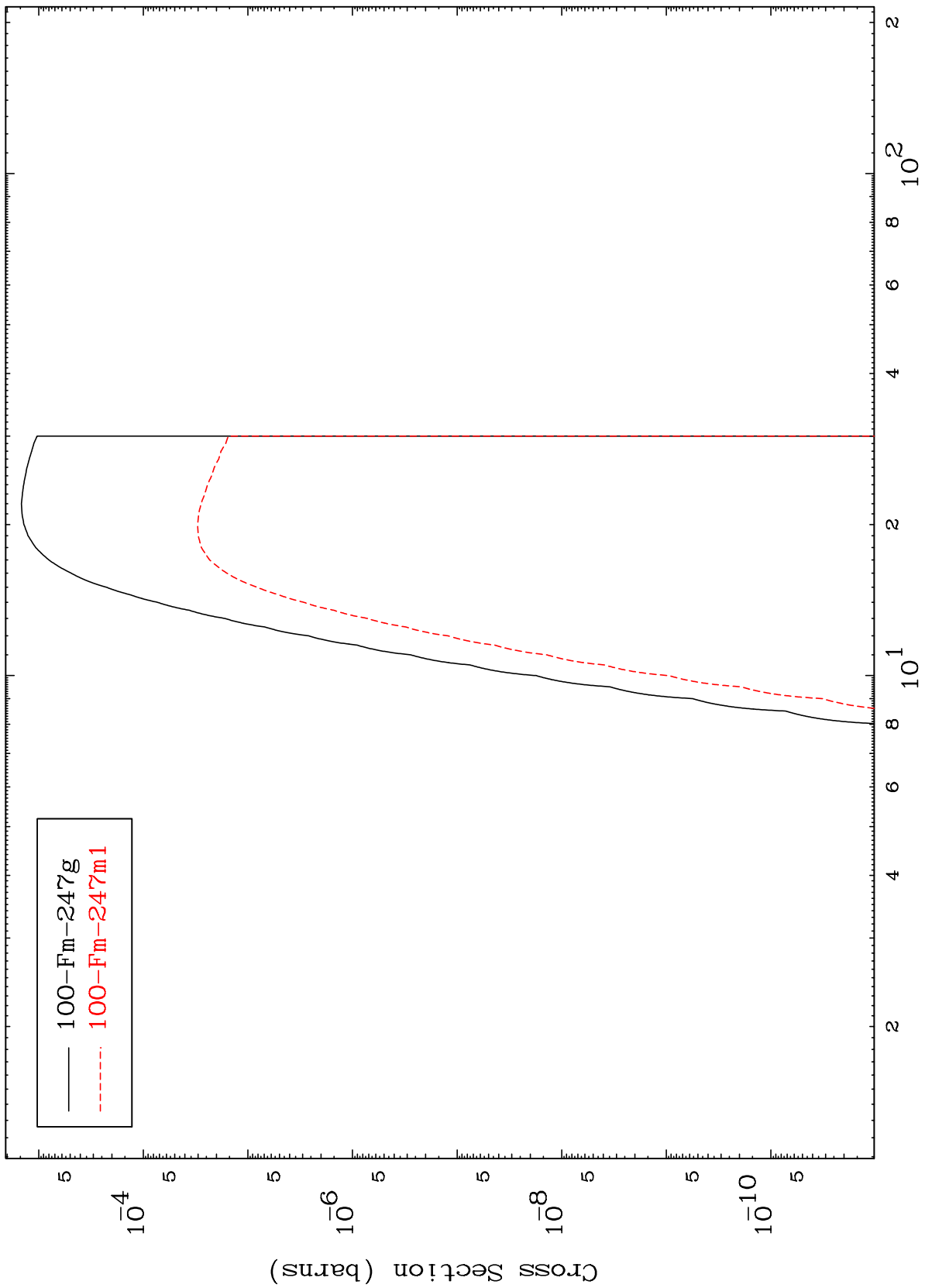
Radionuclide Production Cross Section



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101-Md-249m

(n, t)
Radionuclide Production Cross Section



101-Md-249m

Incident Energy (MeV)

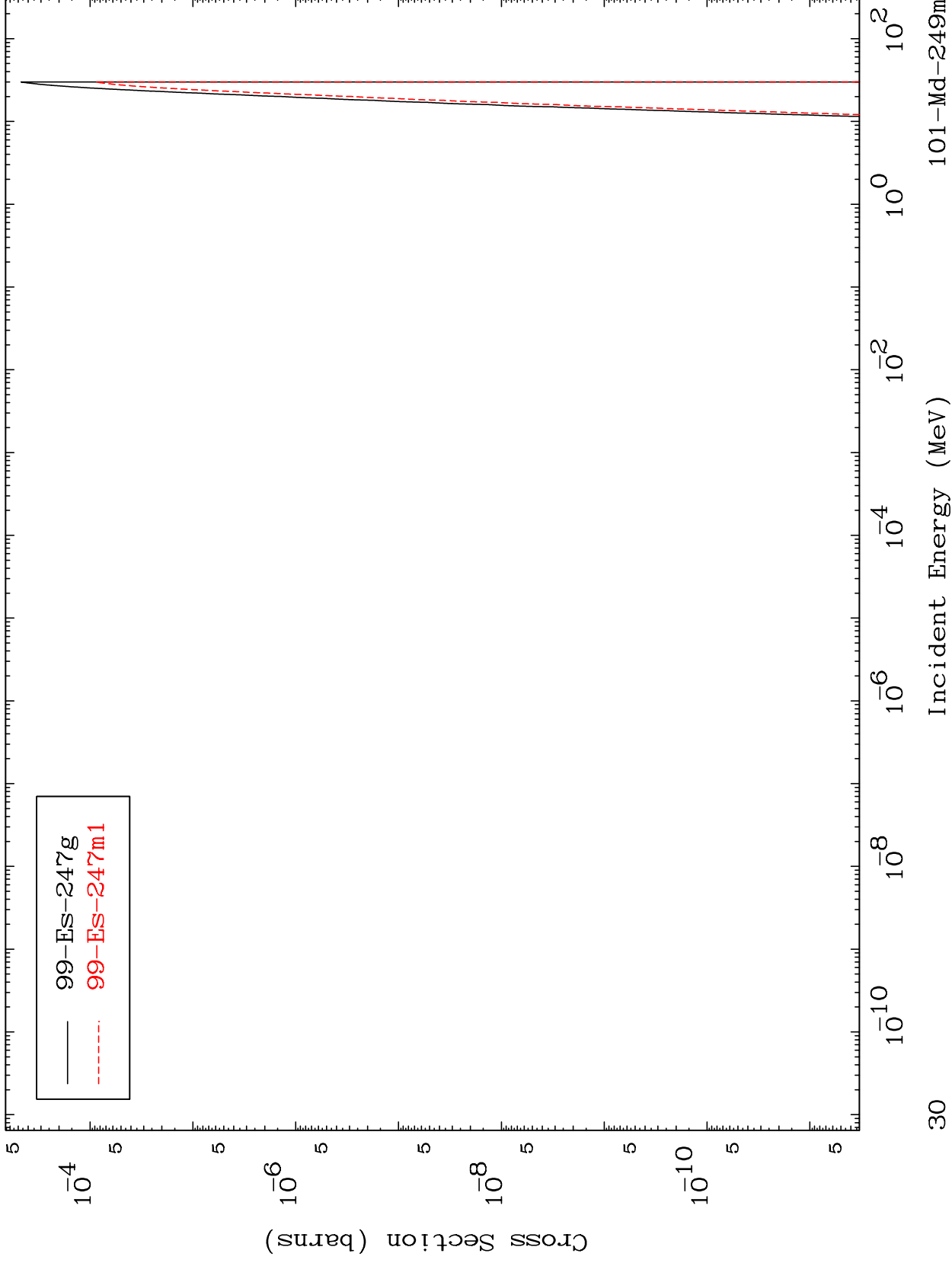
29

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(n,He-3)

101-Md-249m

Radionuclide Production Cross Section



— 99-Es-247g
- - - 99-Es-247m1

MAT 9955

(n,p) d

101-Md-249m

Radionuclide Production Cross Section

