

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

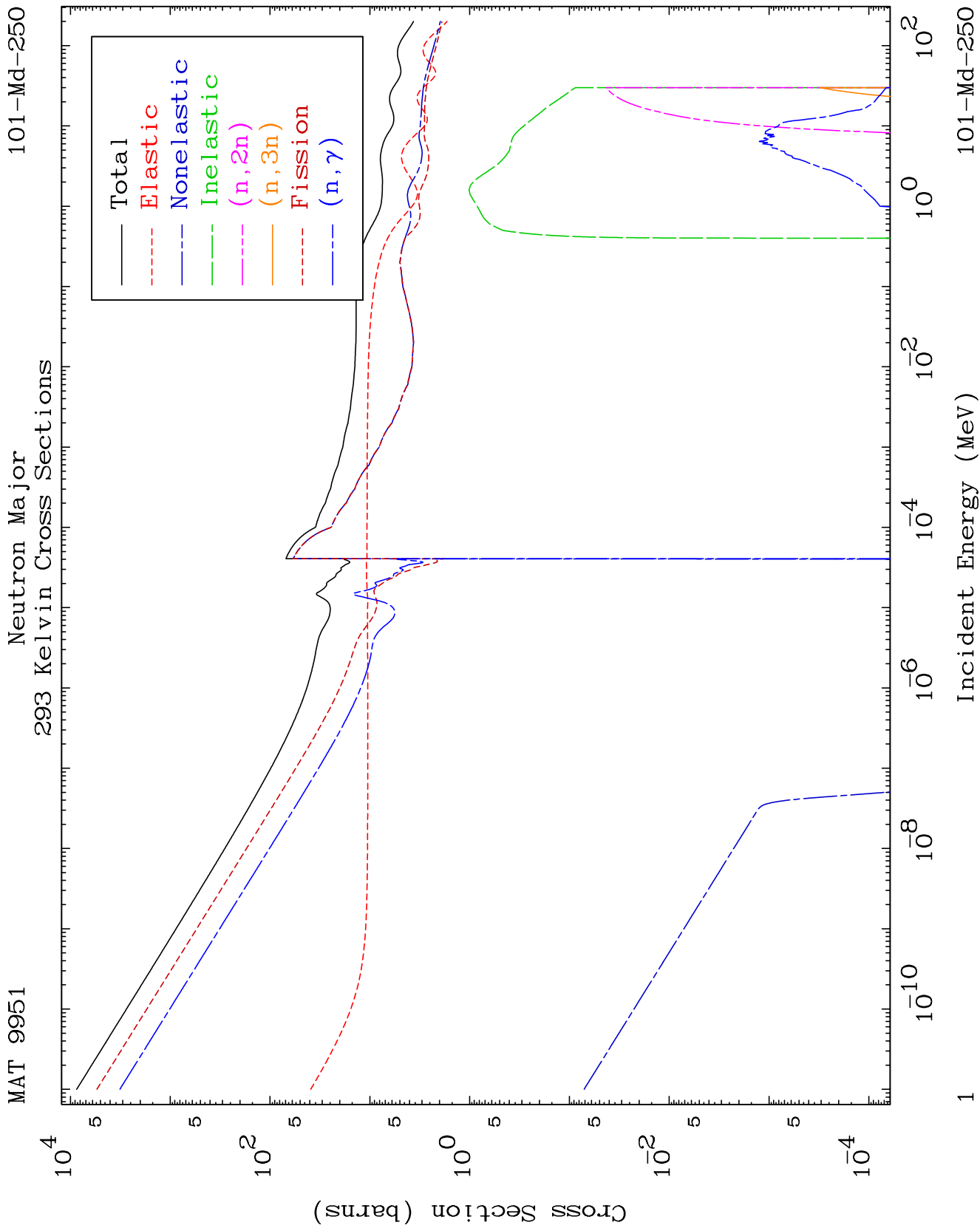
Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

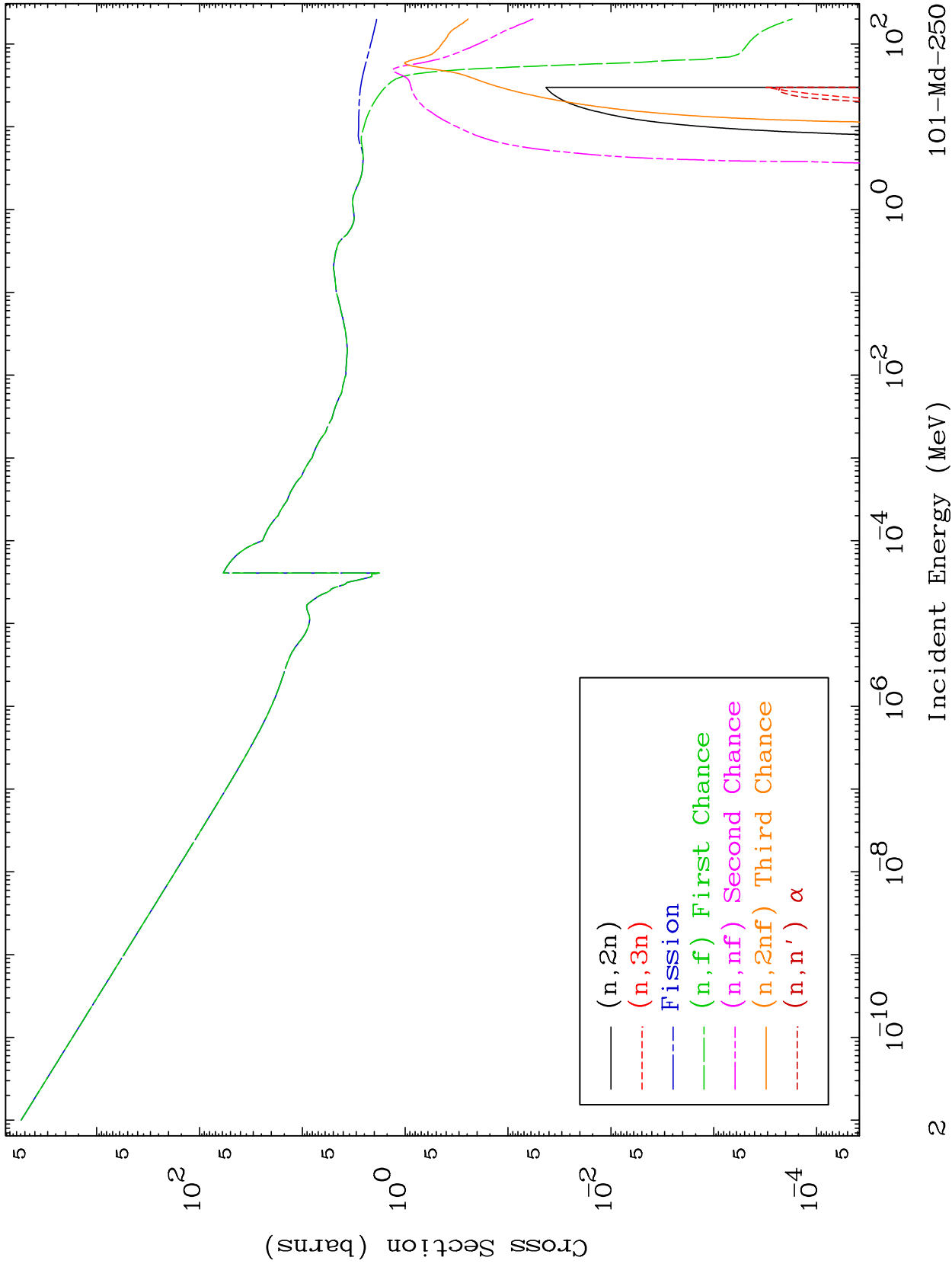
Press Mouse Button to Start



MAT 9951

Neutron Absorption  
293 Kelvin Cross Sections

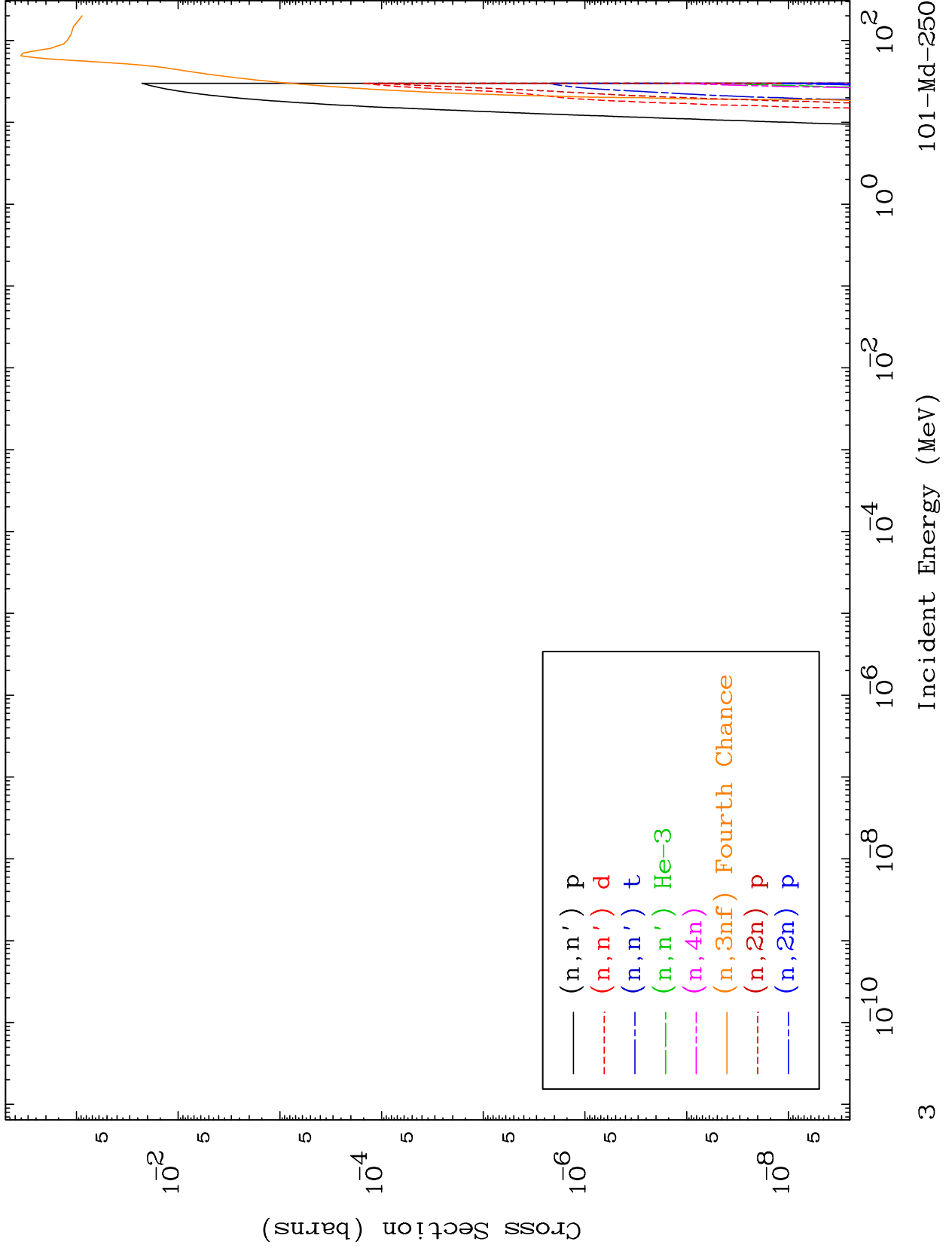
101-Md-250



MAT 9951

Neutron Absorption  
293 Kelvin Cross Sections

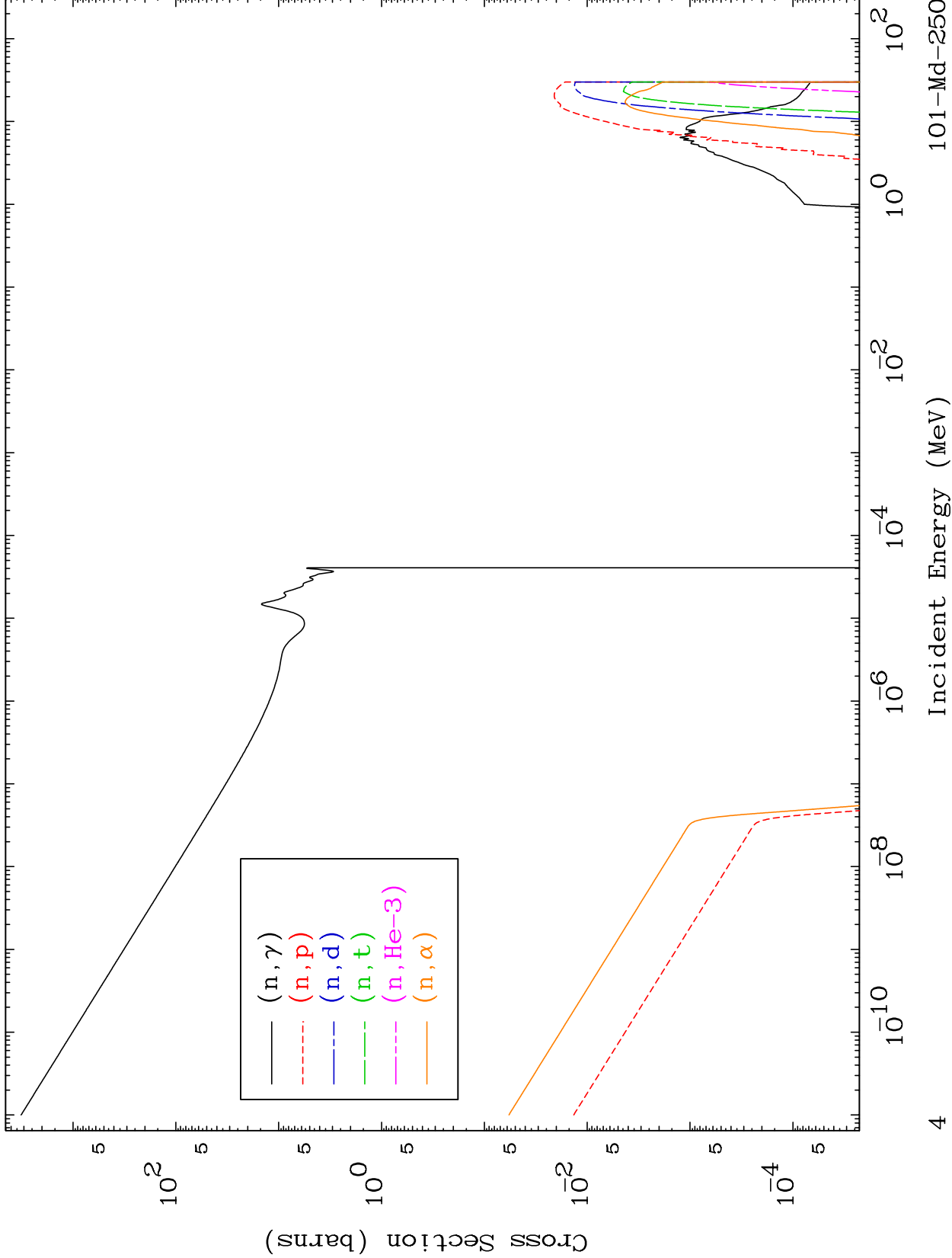
101-Md-250

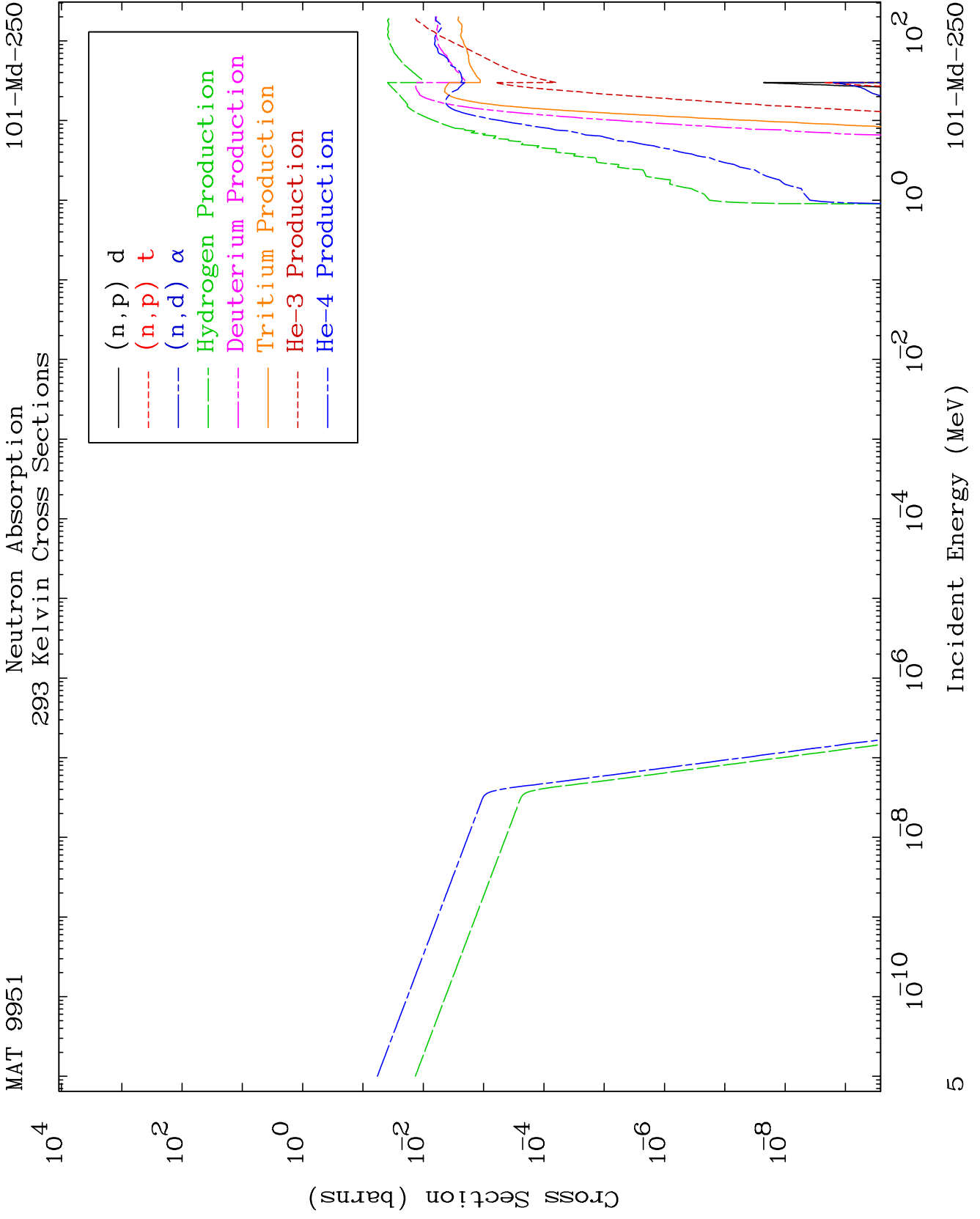


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Neutron Absorption  
293 Kelvin Cross Sections

101-Md-250

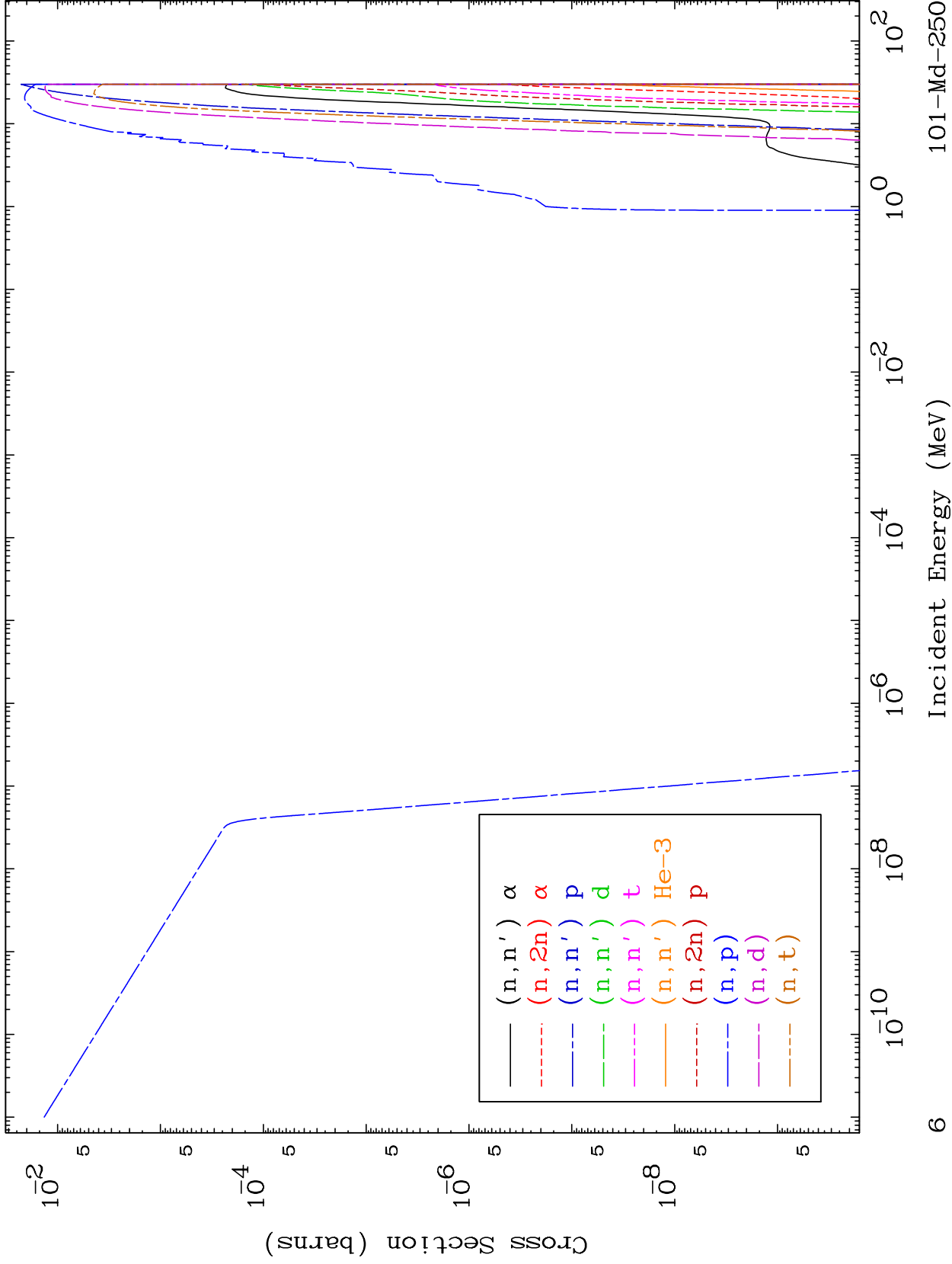




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Charged Particle  
293 Kelvin Cross Sections

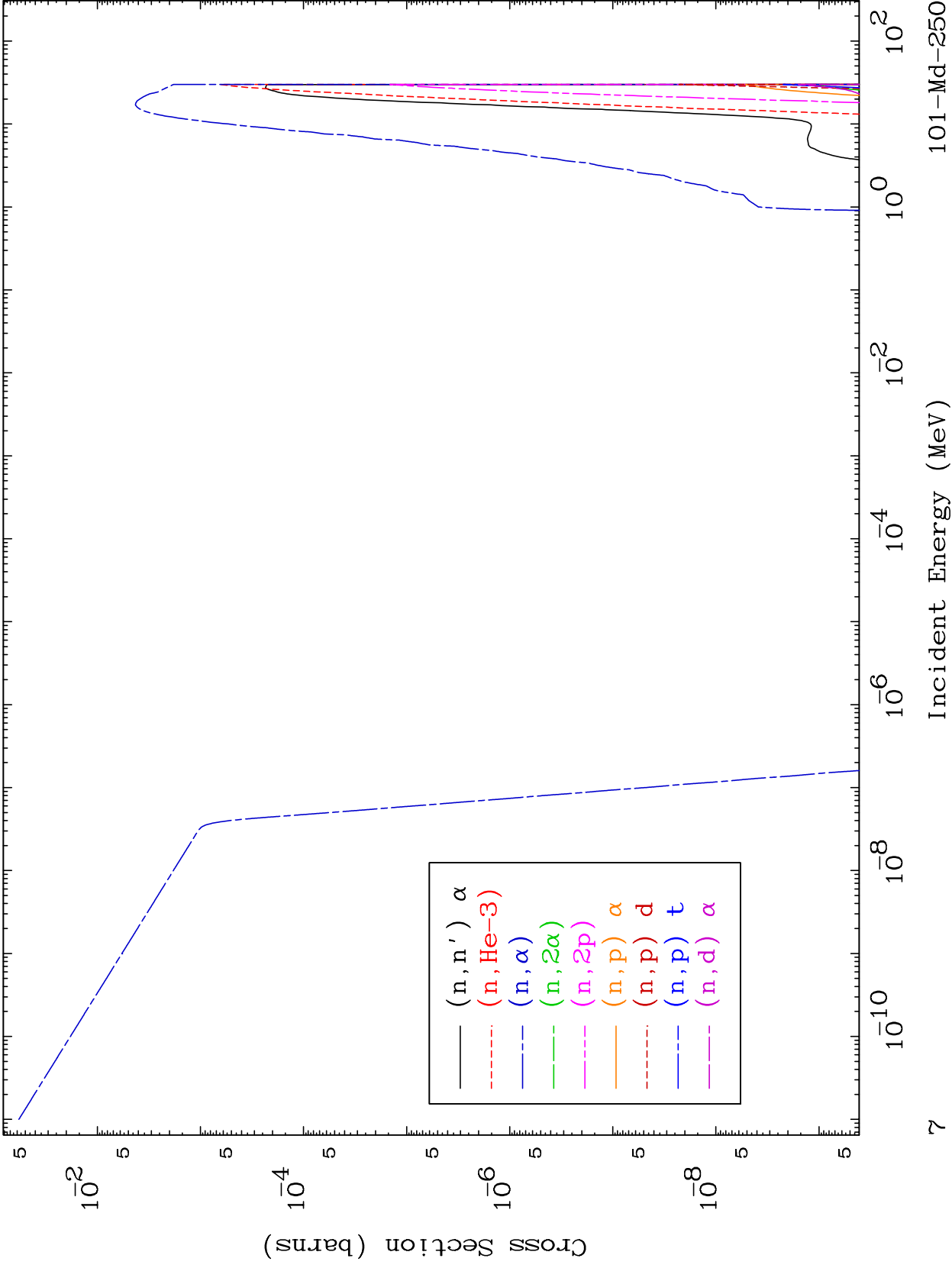
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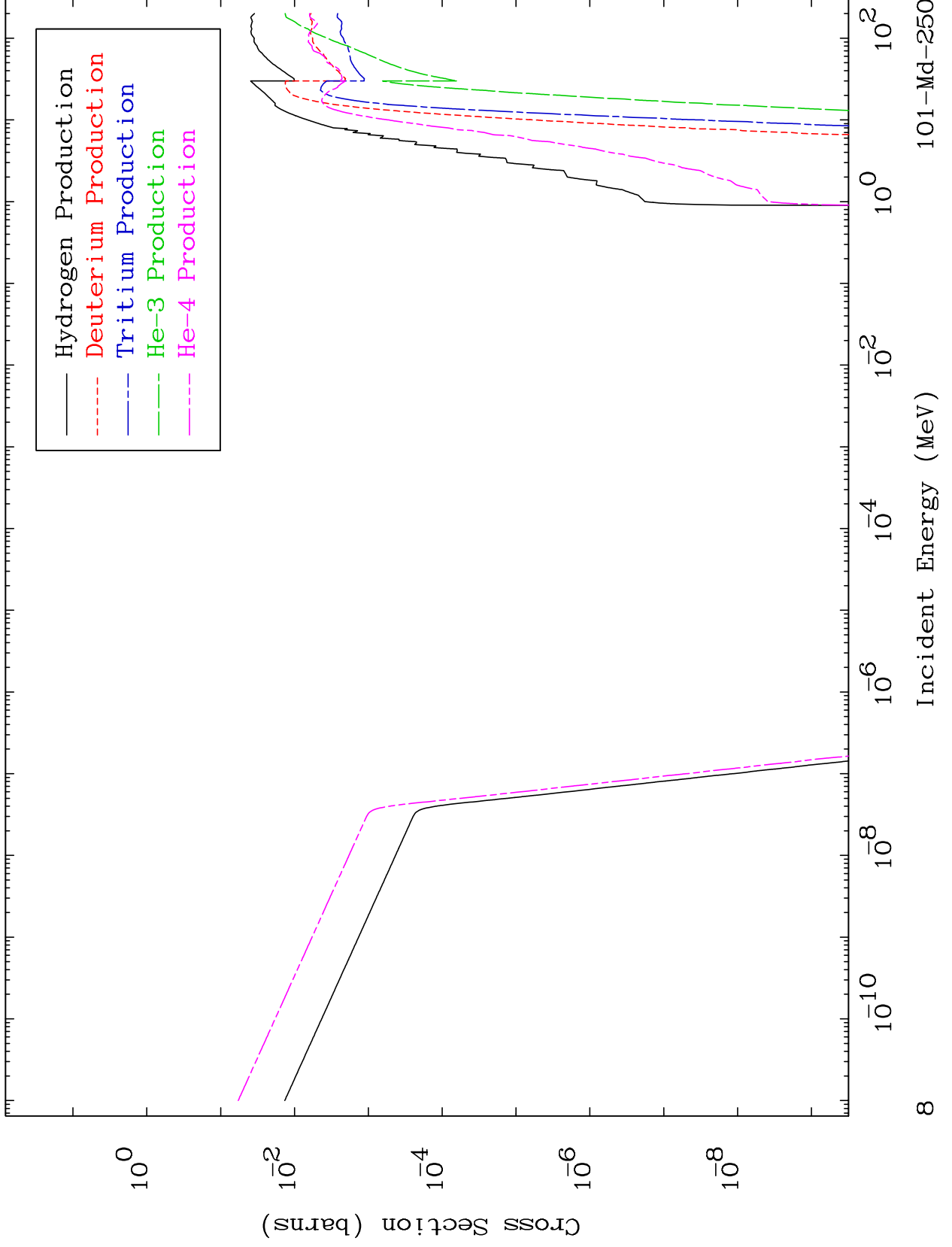


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Charged Particle  
293 Kelvin Cross Sections

101-Md-250

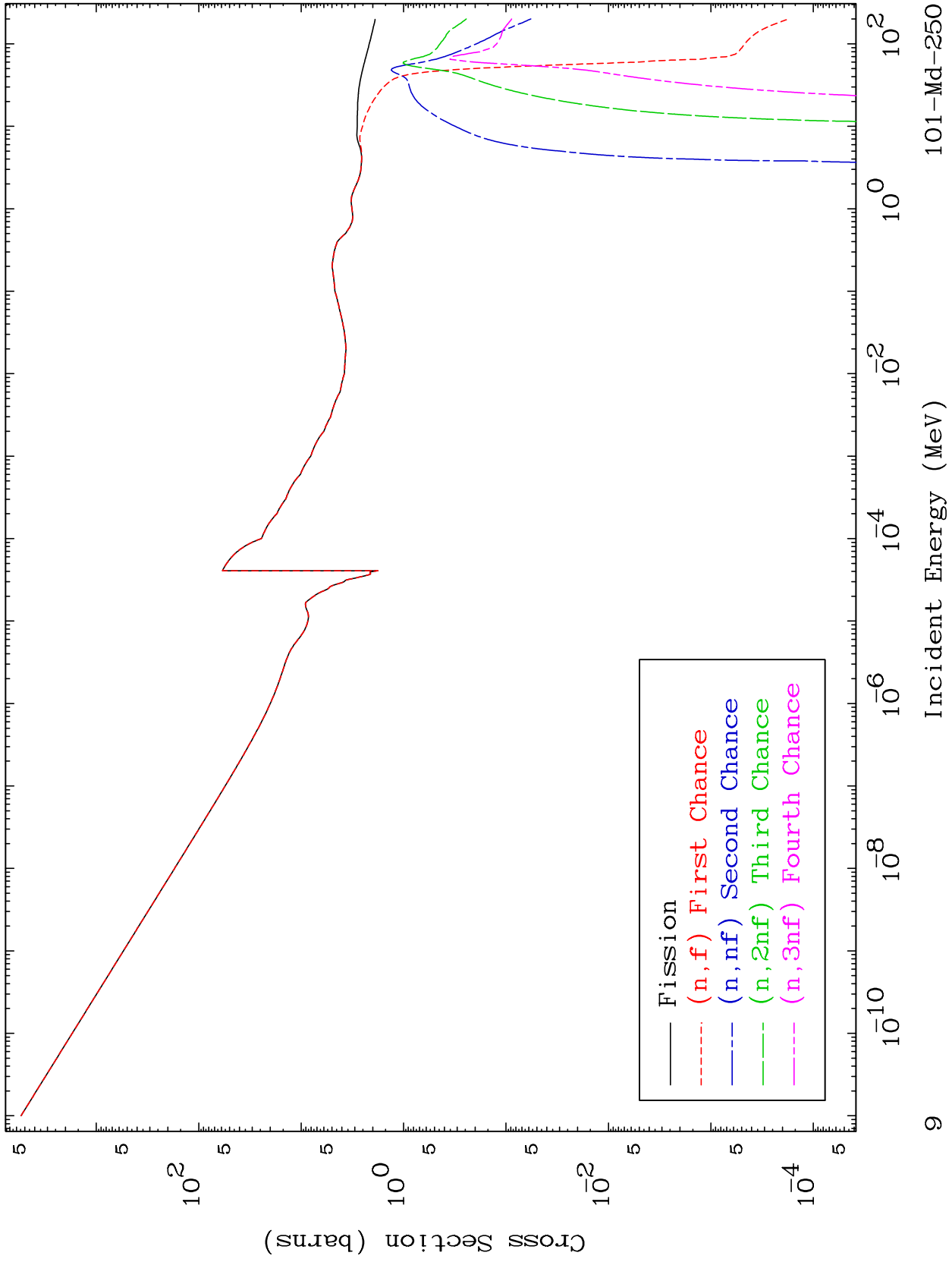


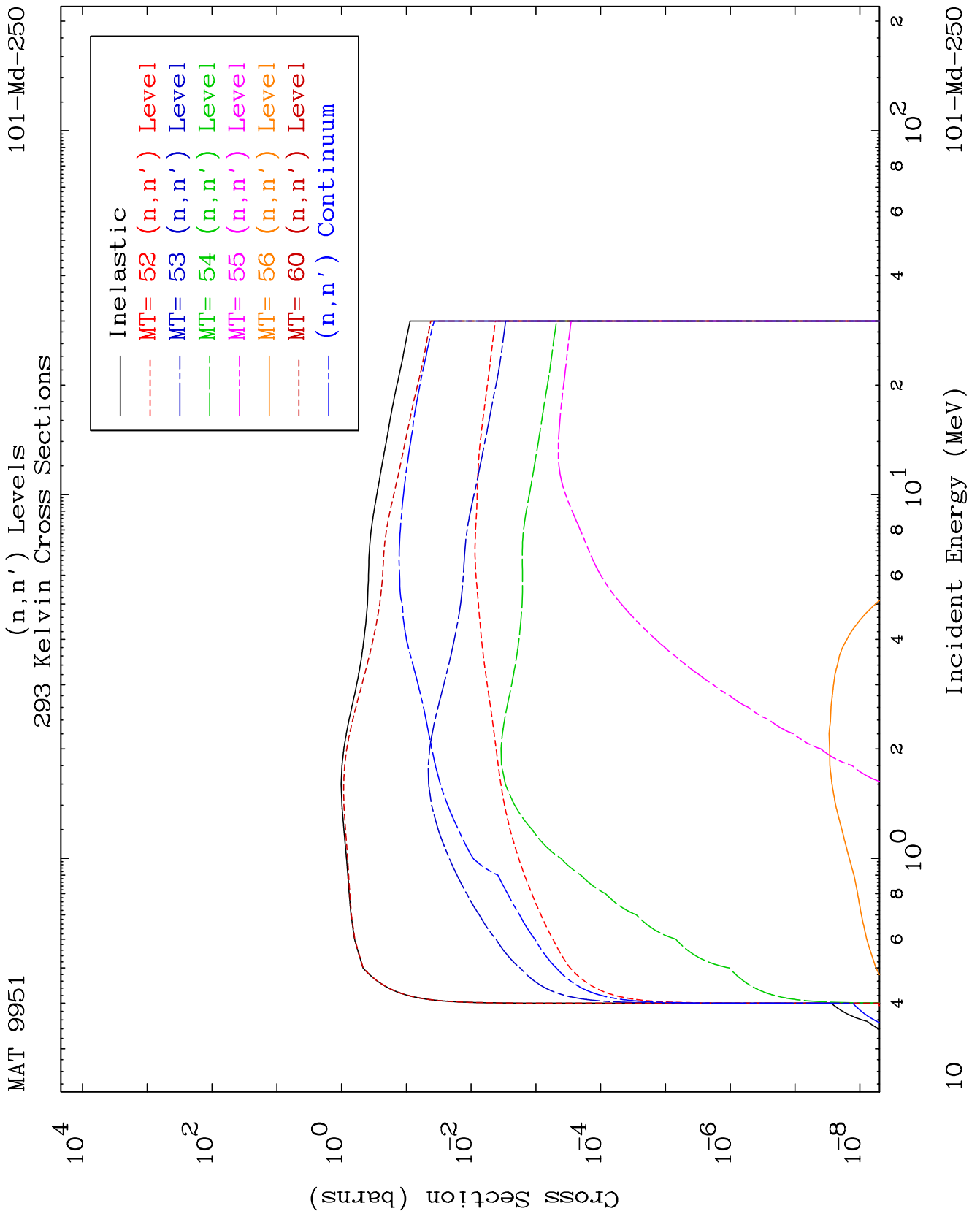


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Fission  
293 Kelvin Cross Sections

101-Md-250

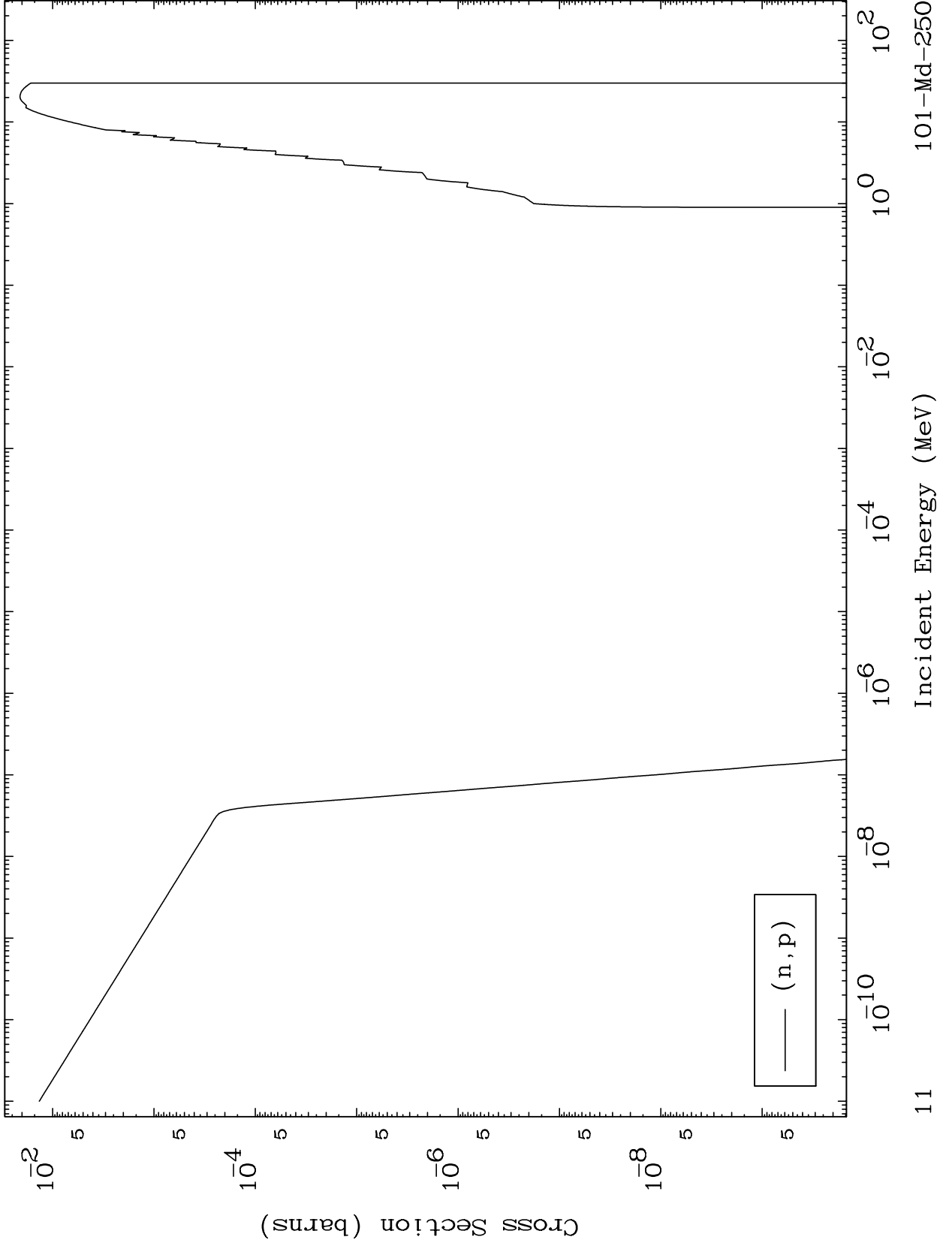




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(n,p) Levels  
293 Kelvin Cross Sections

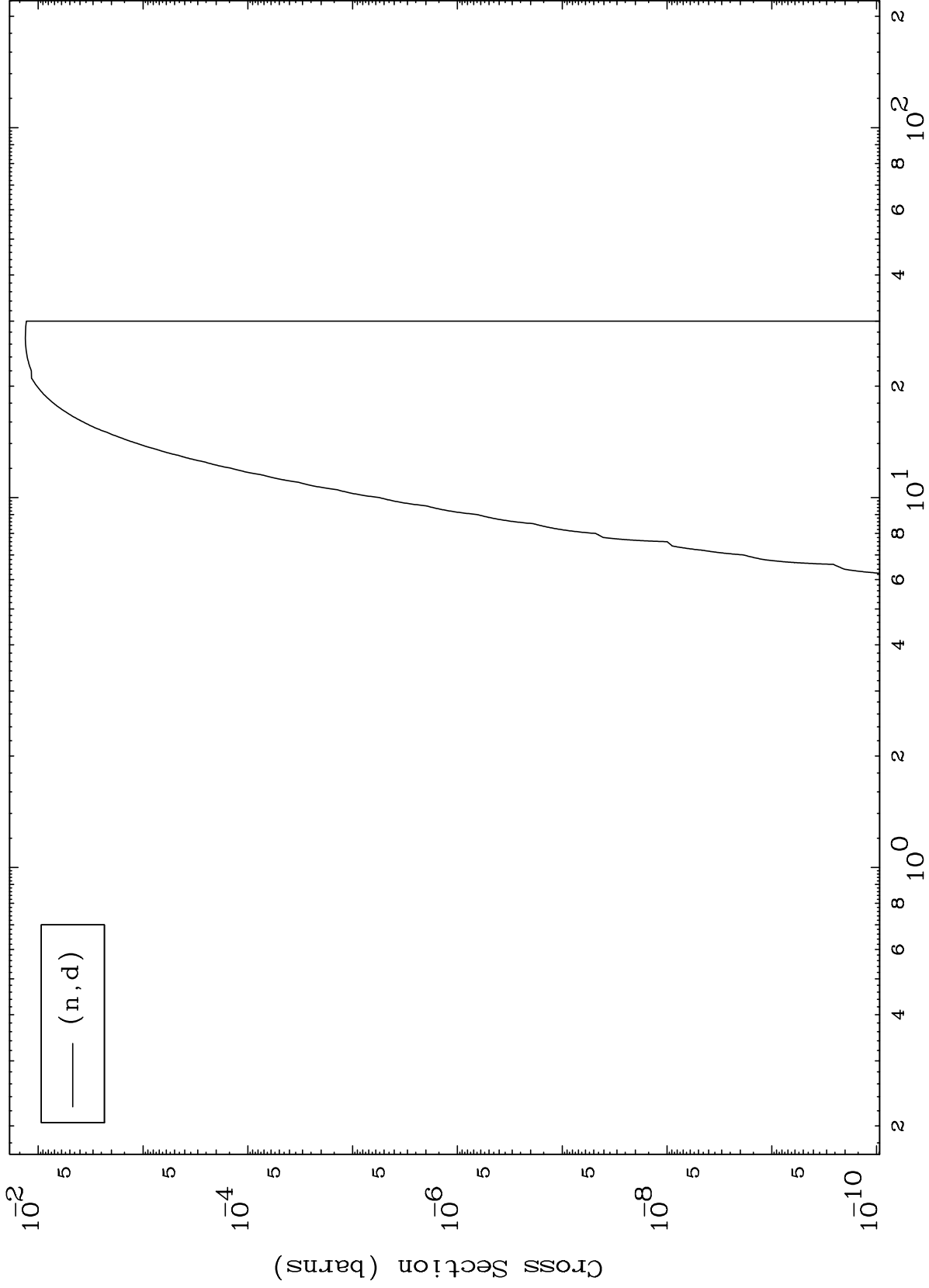
101-Md-250



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(n,d) Levels  
293 Kelvin Cross Sections

101-Md-250



12

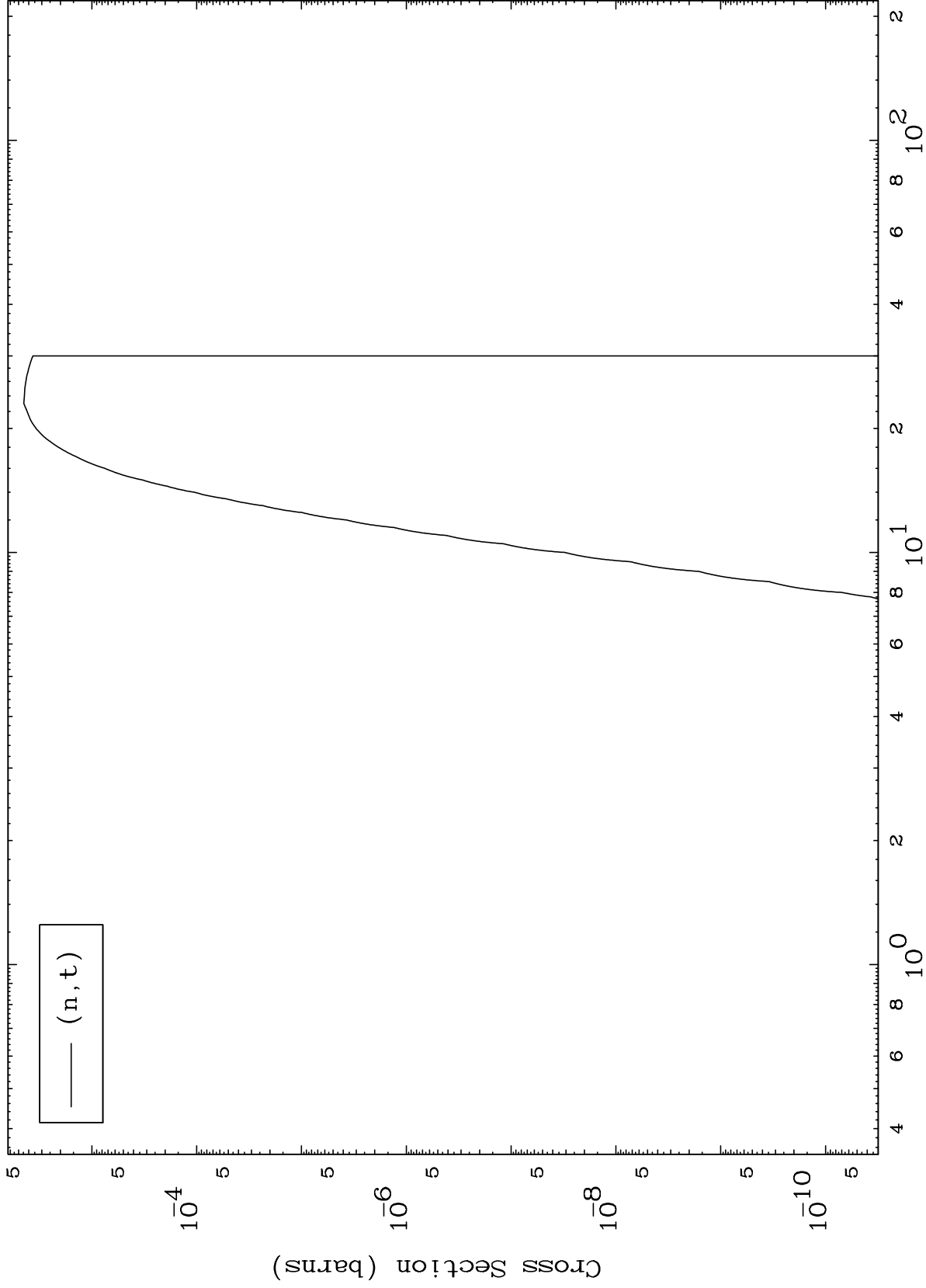
Incident Energy (MeV)

101-Md-250

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(n,t) Levels  
293 Kelvin Cross Sections

101-Md-250



13

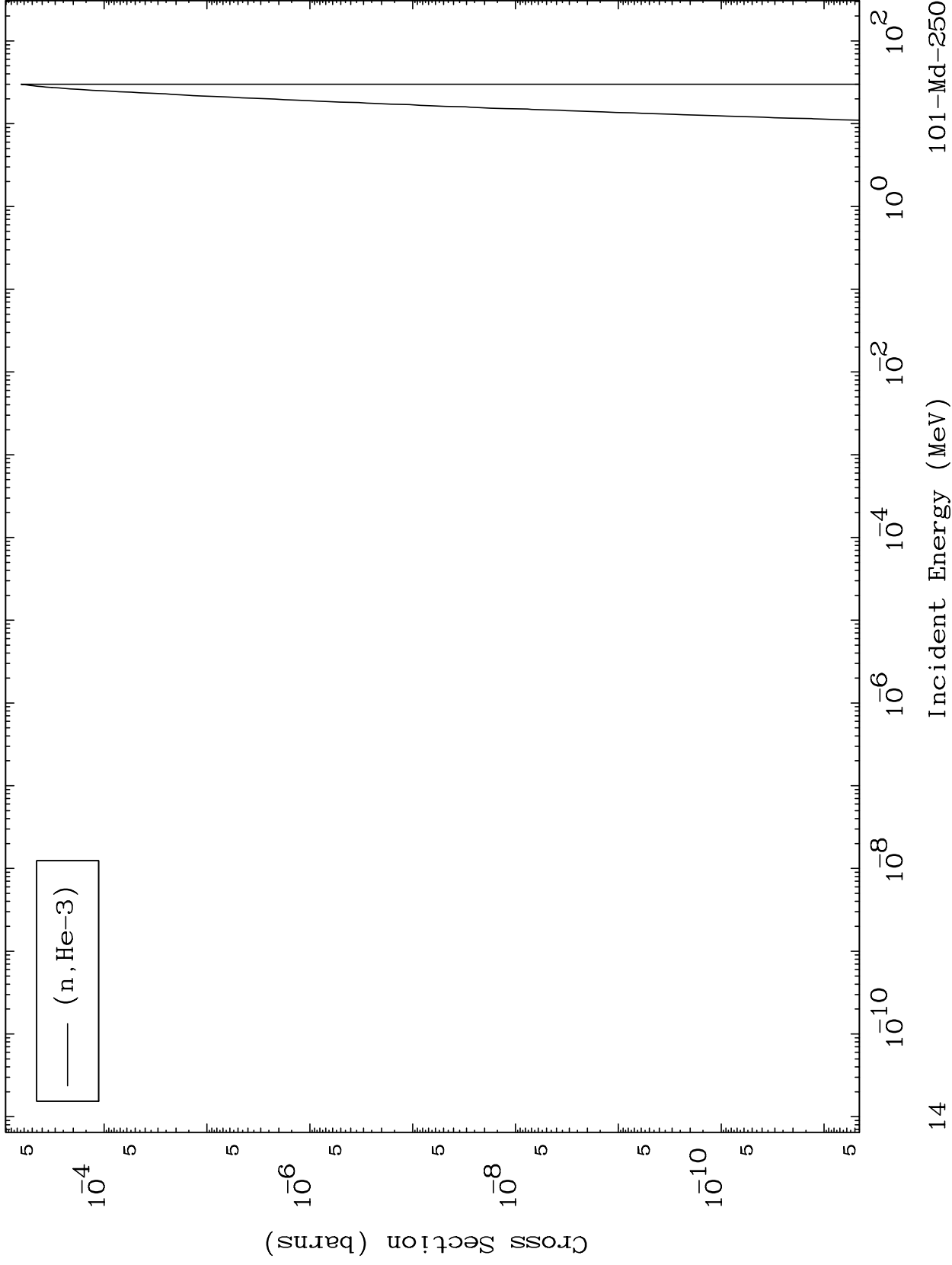
Incident Energy (MeV)

101-Md-250

MAT 9951

(n,He3) Levels  
293 Kelvin Cross Sections

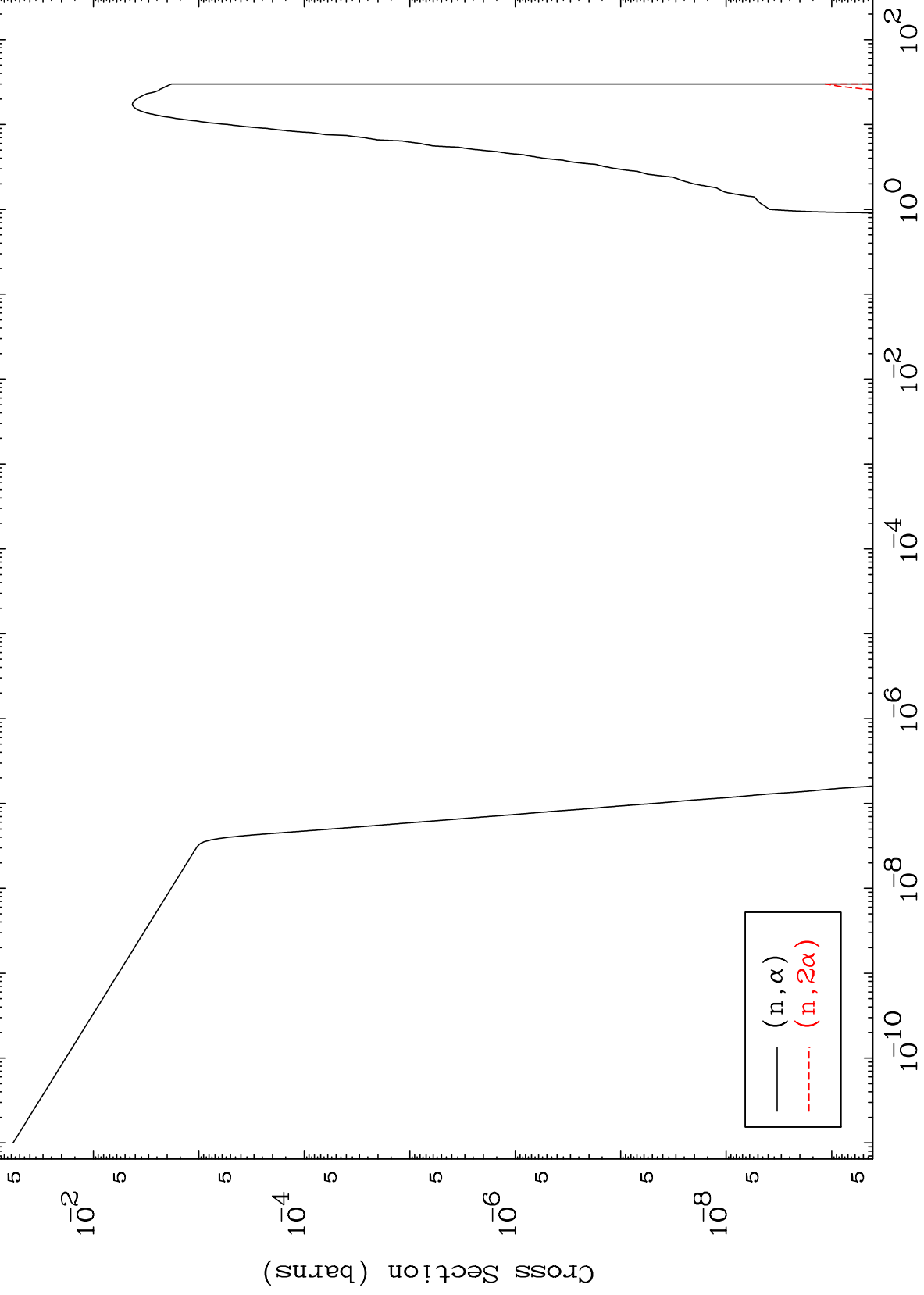
101-Md-250



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(n,α) Levels  
293 Kelvin Cross Sections

101-Md-250



15

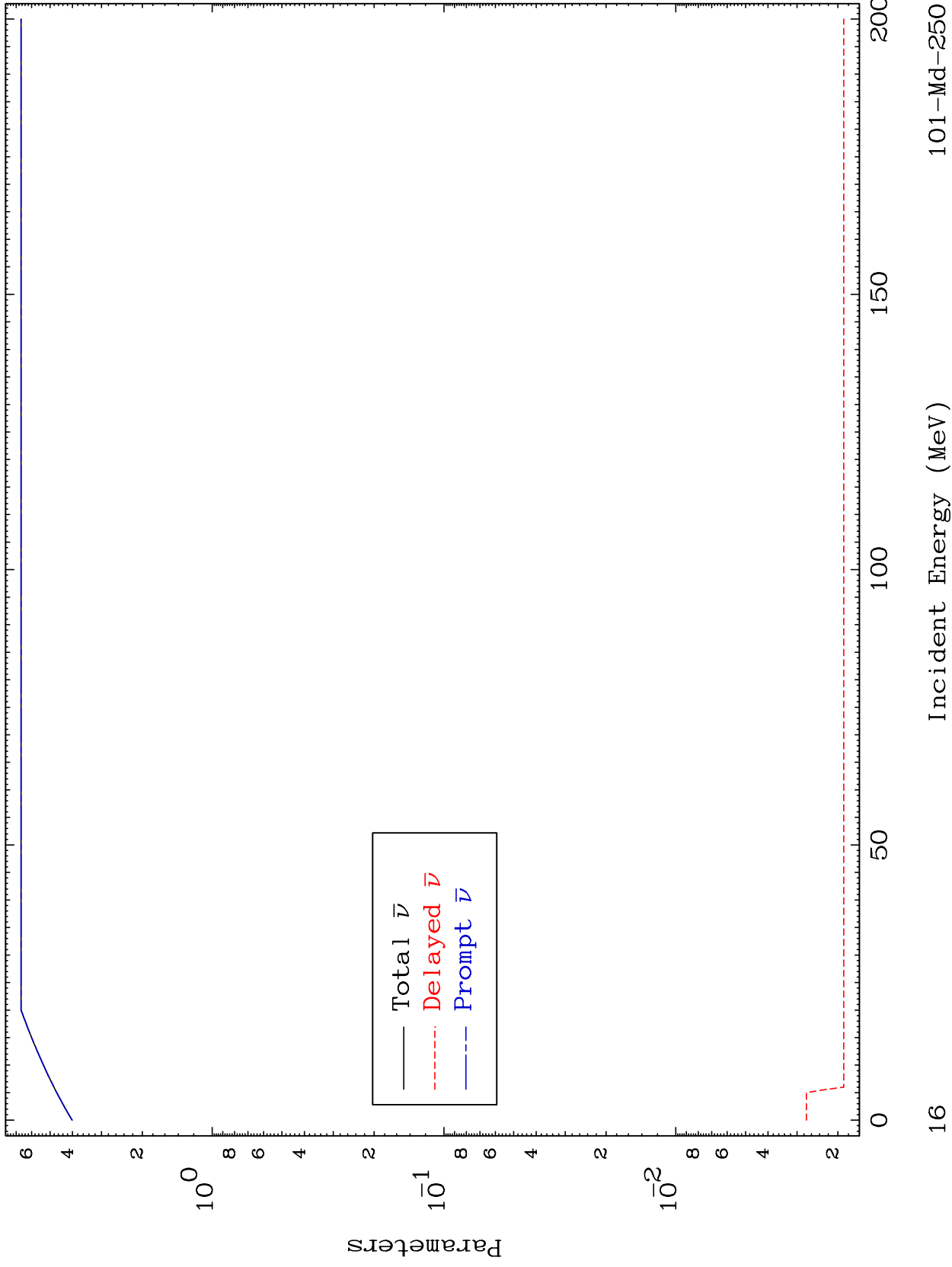
Incident Energy (MeV)

101-Md-250

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Energy Release  
Parameters

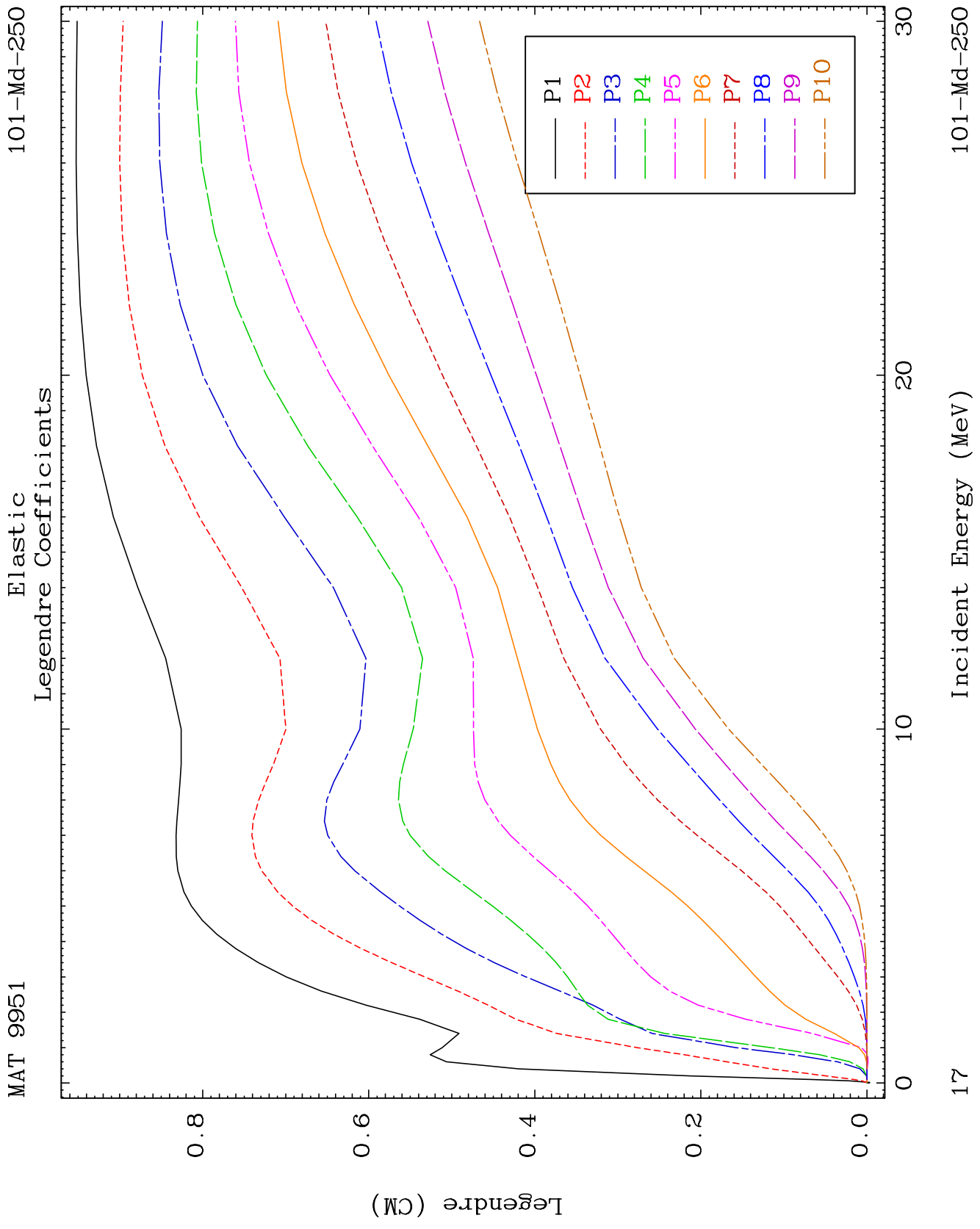
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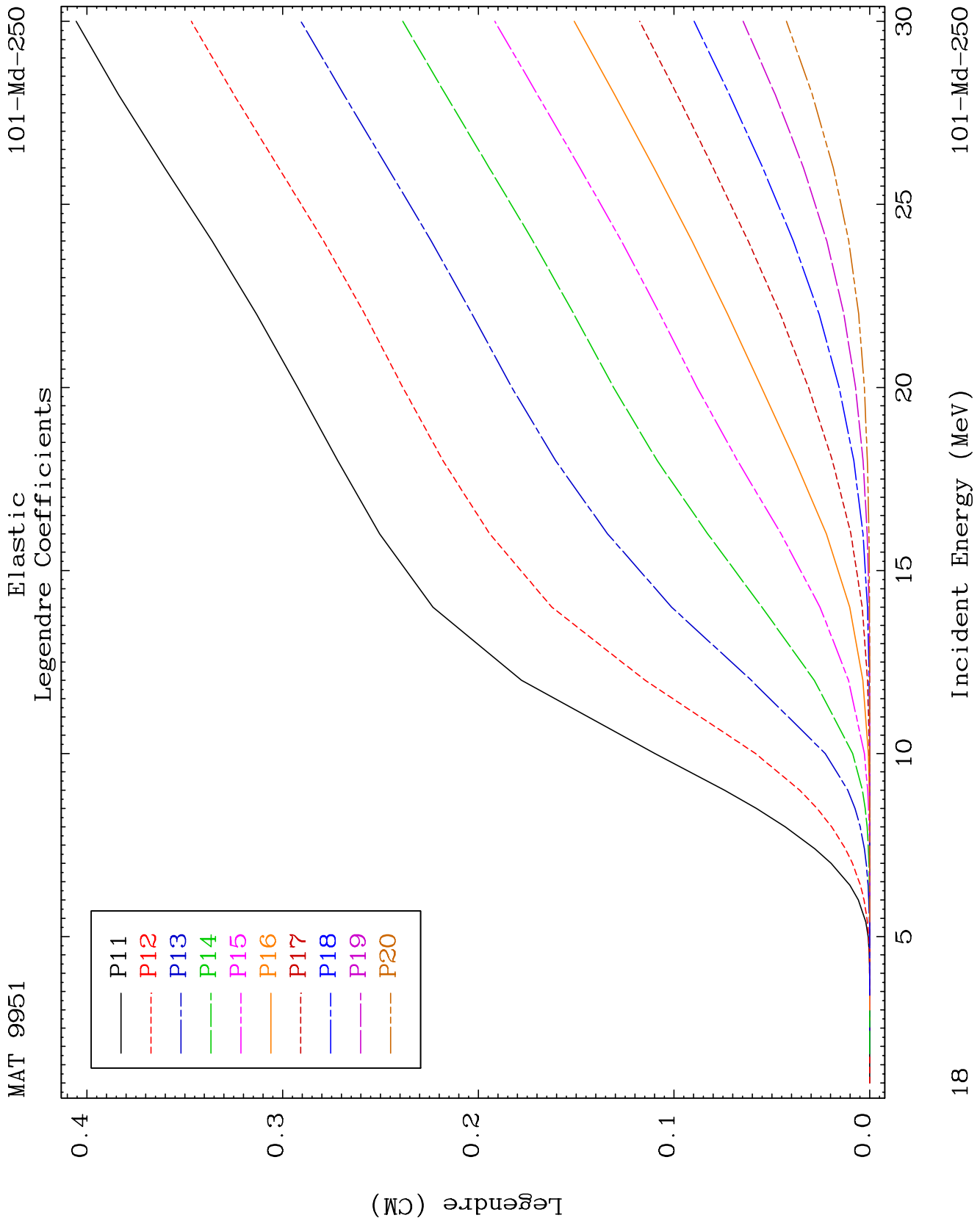


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Incident Energy (MeV)

101-Md-250

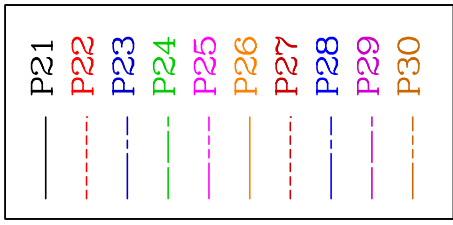
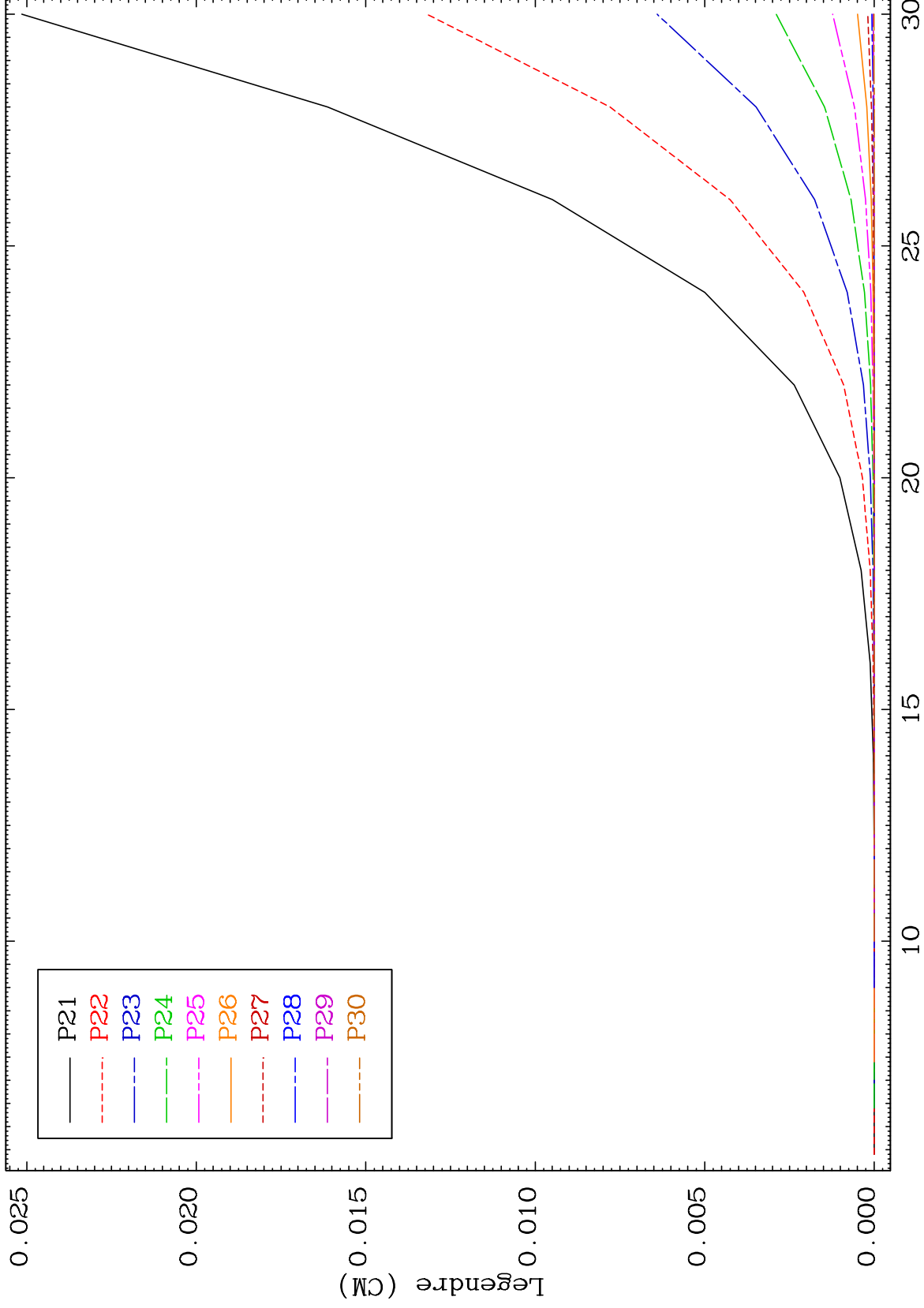




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Elastic  
Legendre Coefficients

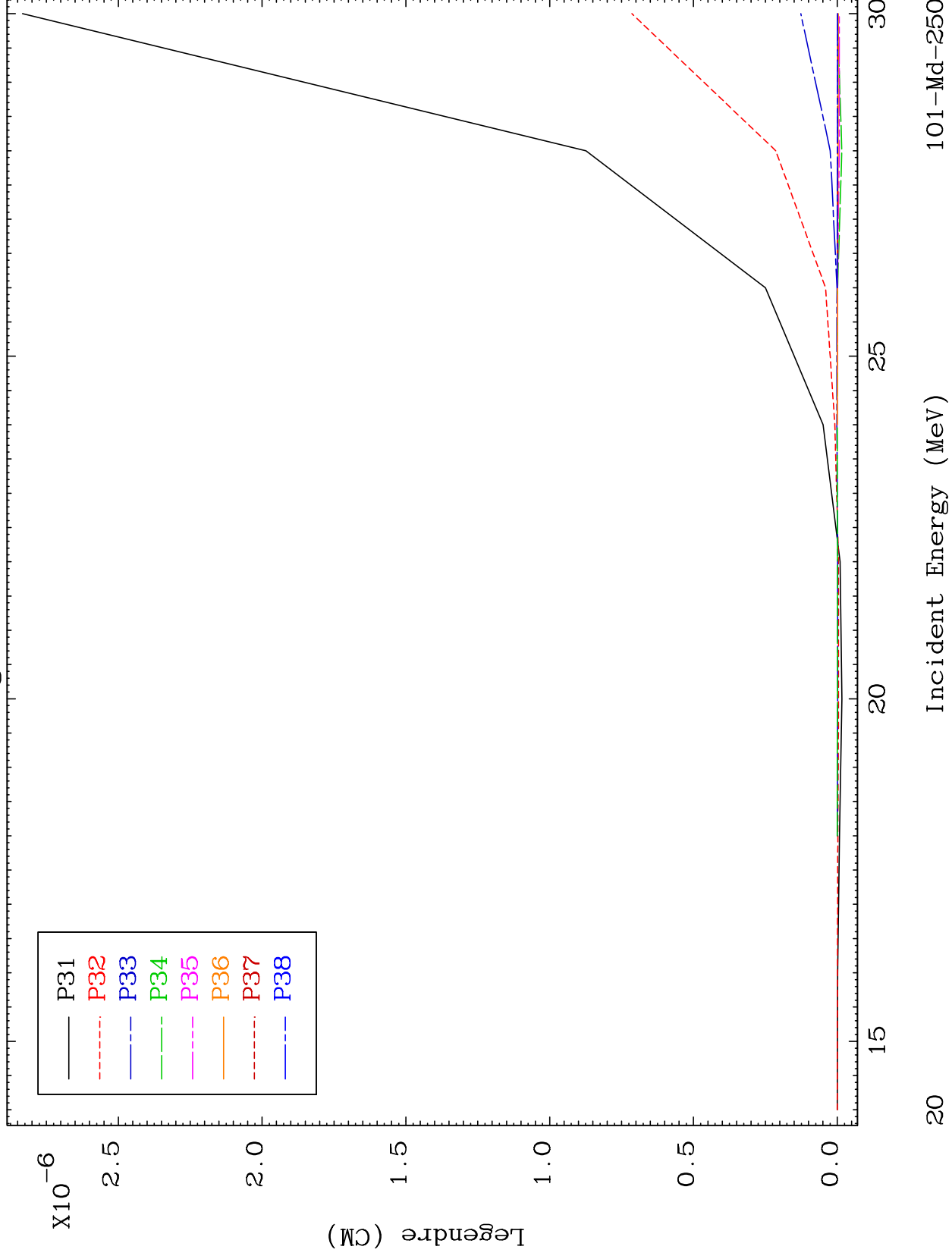
101-Md-250



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Elastic  
Legendre Coefficients

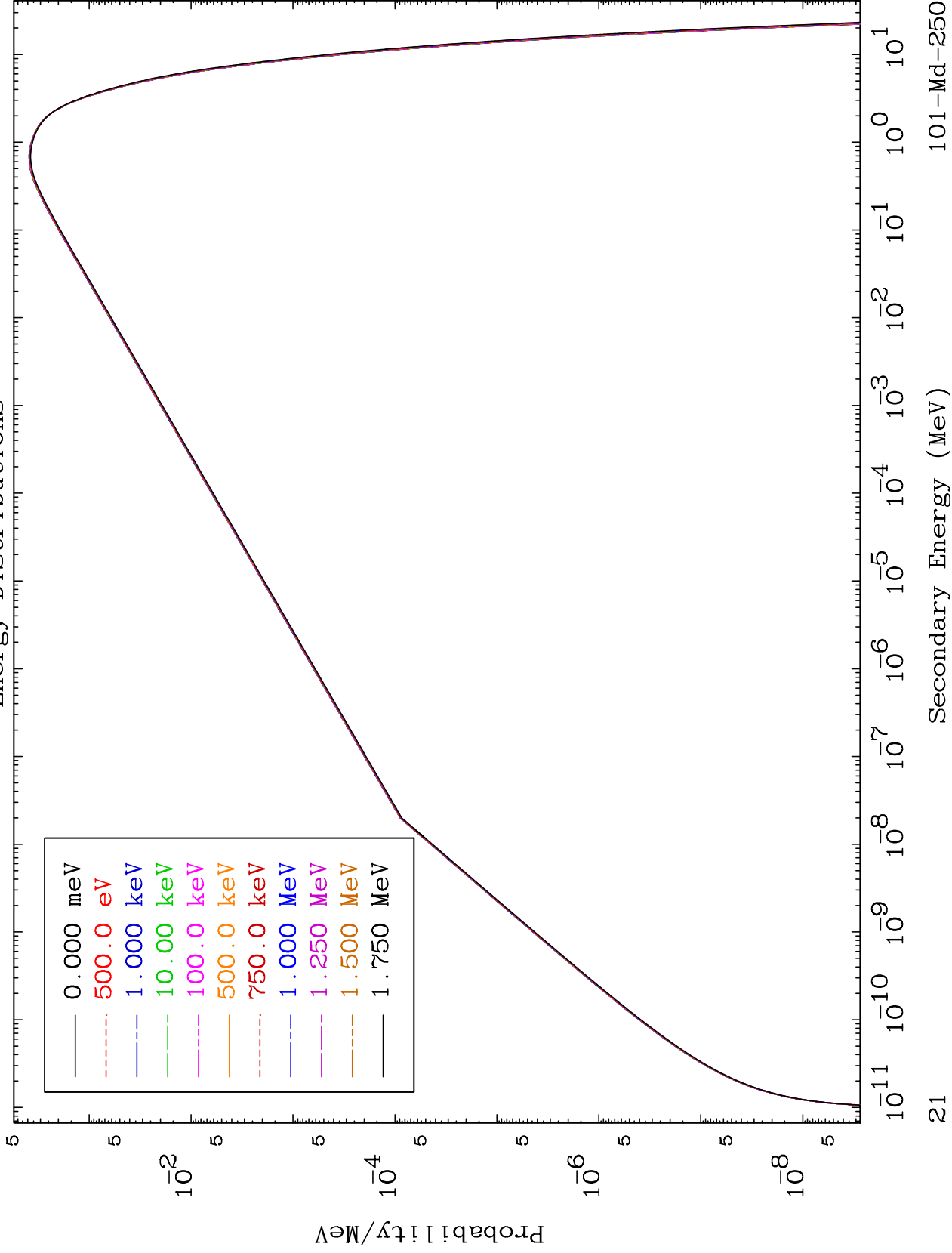
101-Md-250



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### Fission Energy Distributions

101-Md-250

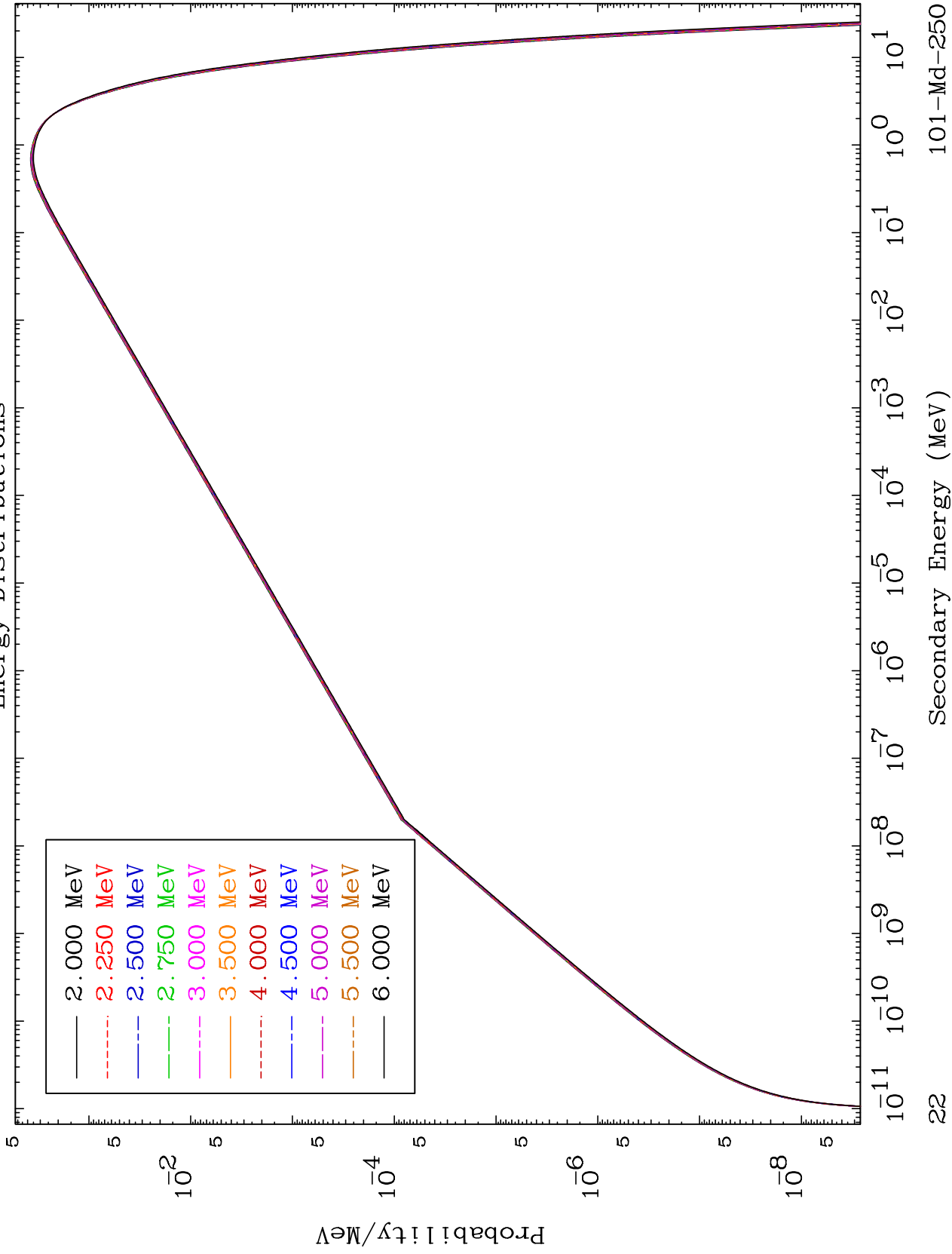


101-Md-250

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### Fission Energy Distributions

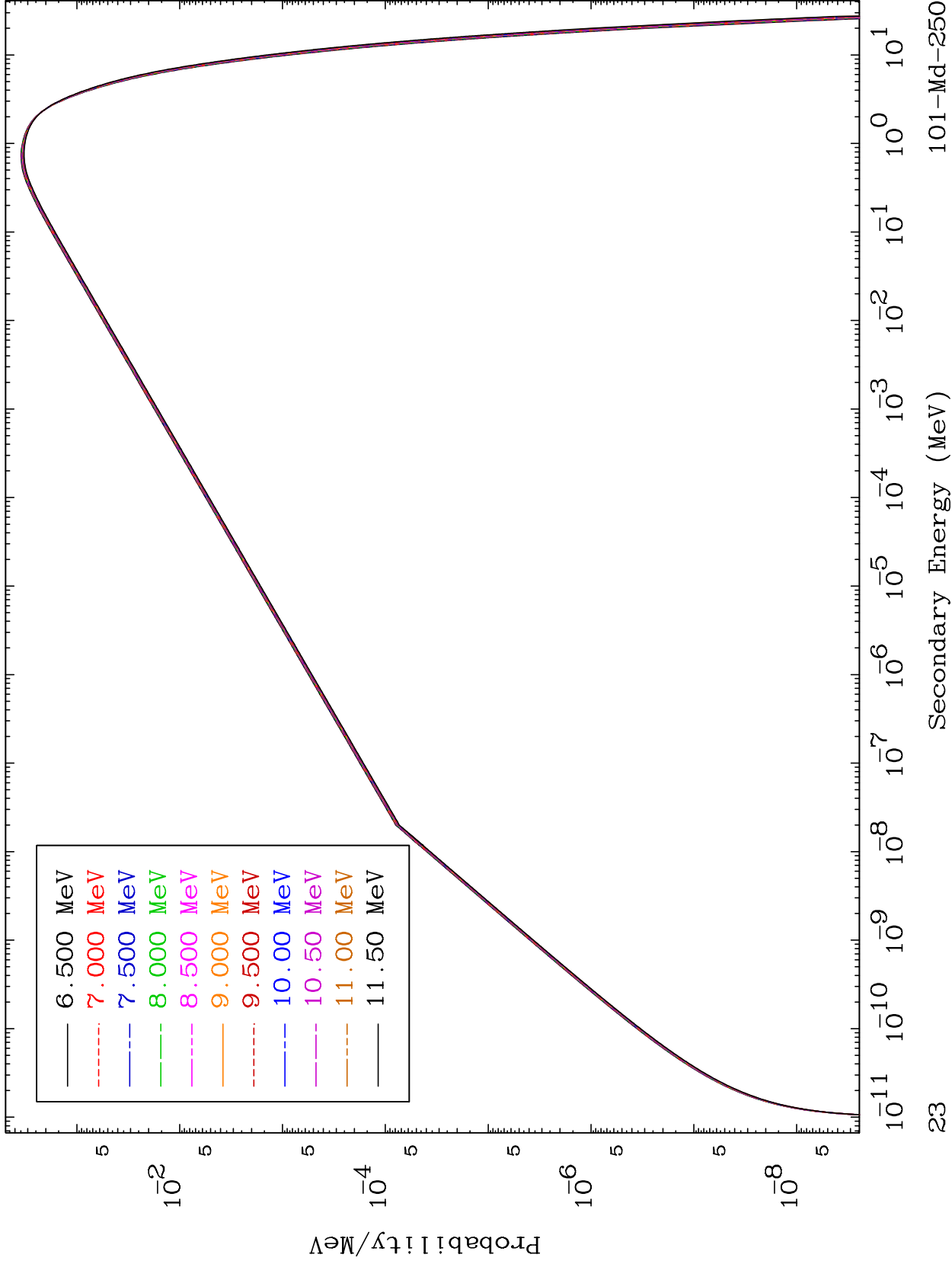
101-Md-250



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Fission  
Energy Distributions

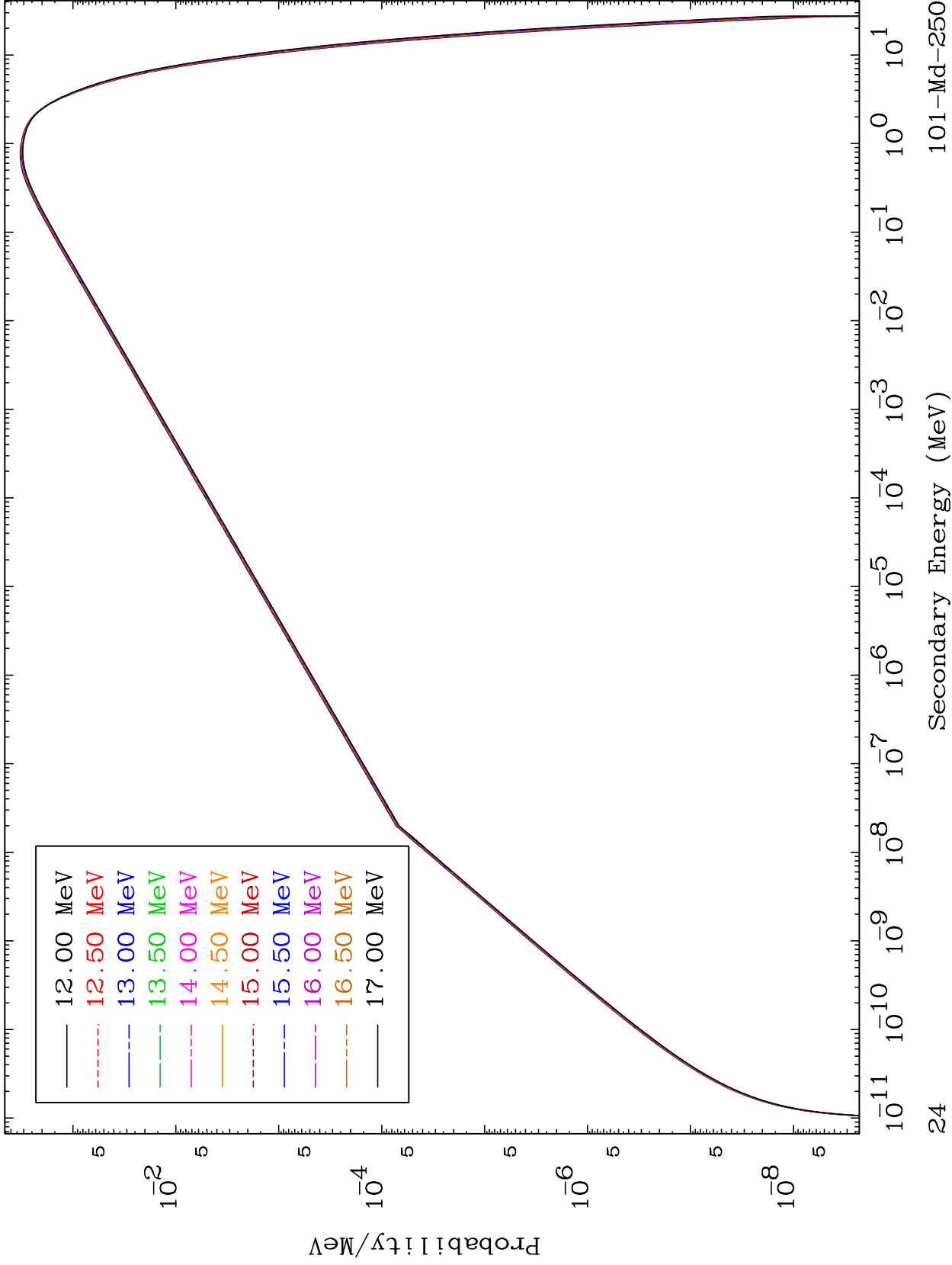
101-Md-250



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### Fission Energy Distributions

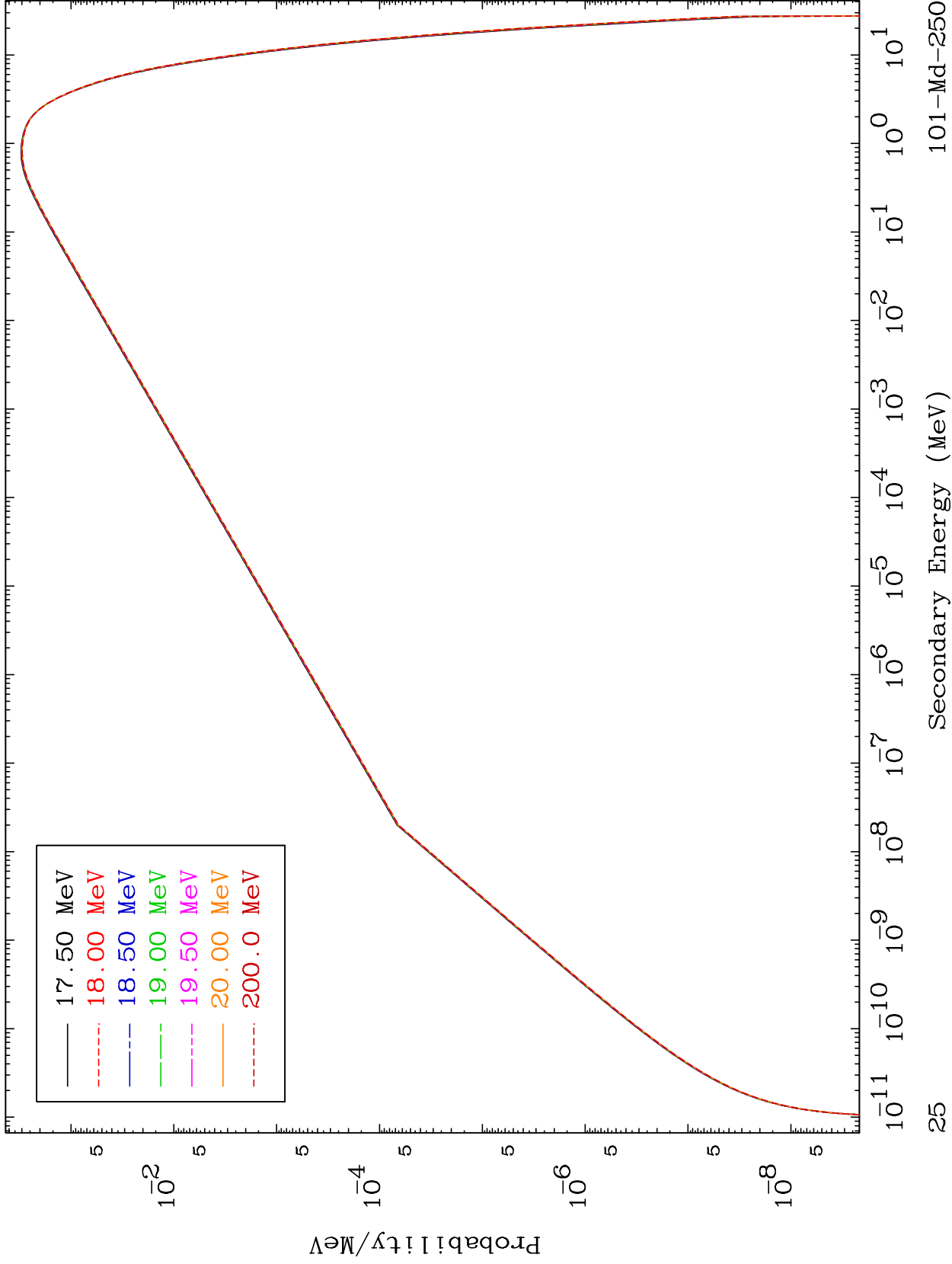
101-Md-250



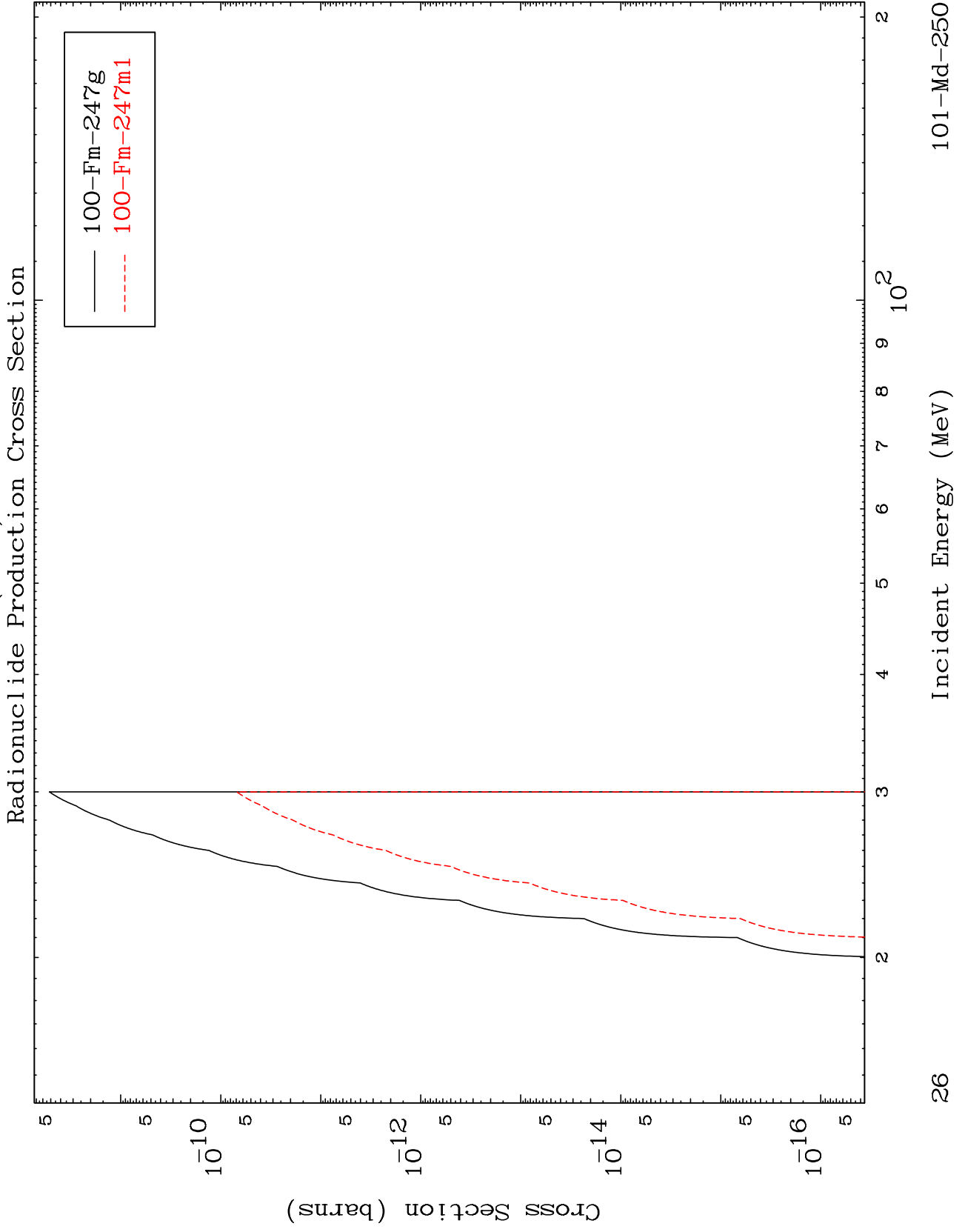
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Fission  
Energy Distributions

101-Md-250



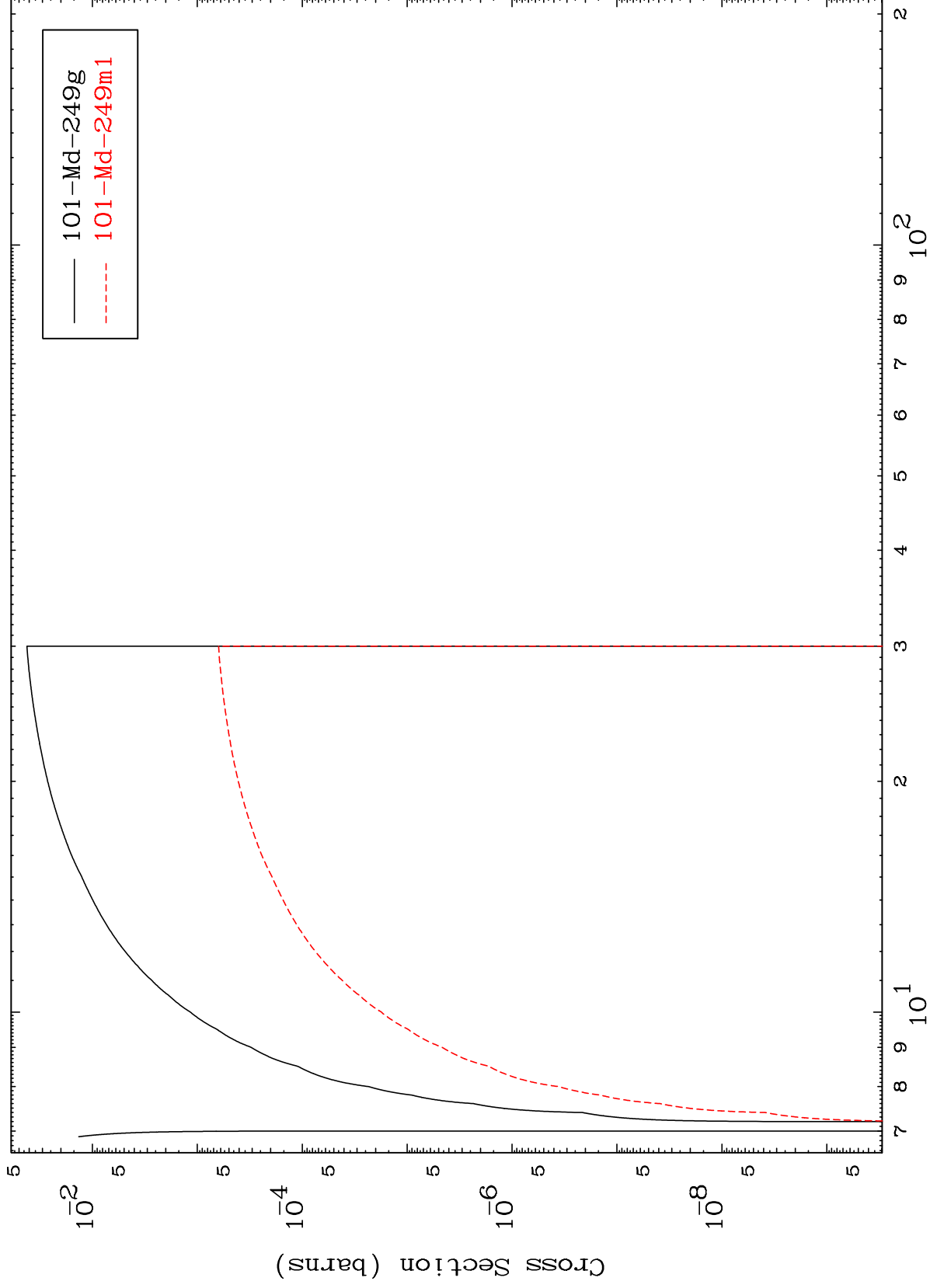
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101-Md-250

(n,2n)  
Radionuclide Production Cross Section



101-Md-250

Incident Energy (MeV)

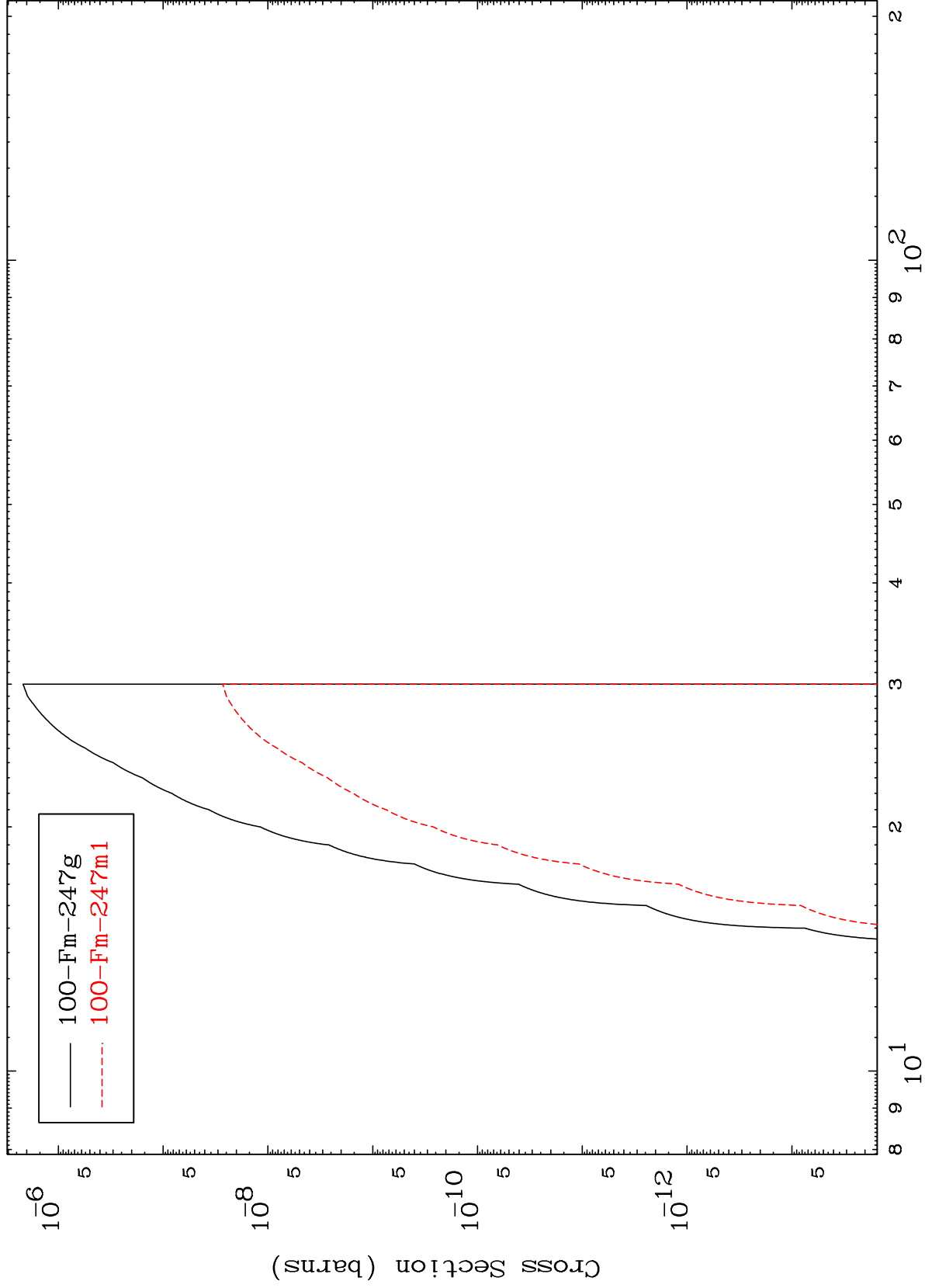
27

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(n,n') t

101-Md-250

Radionuclide Production Cross Section



28

Incident Energy (MeV)

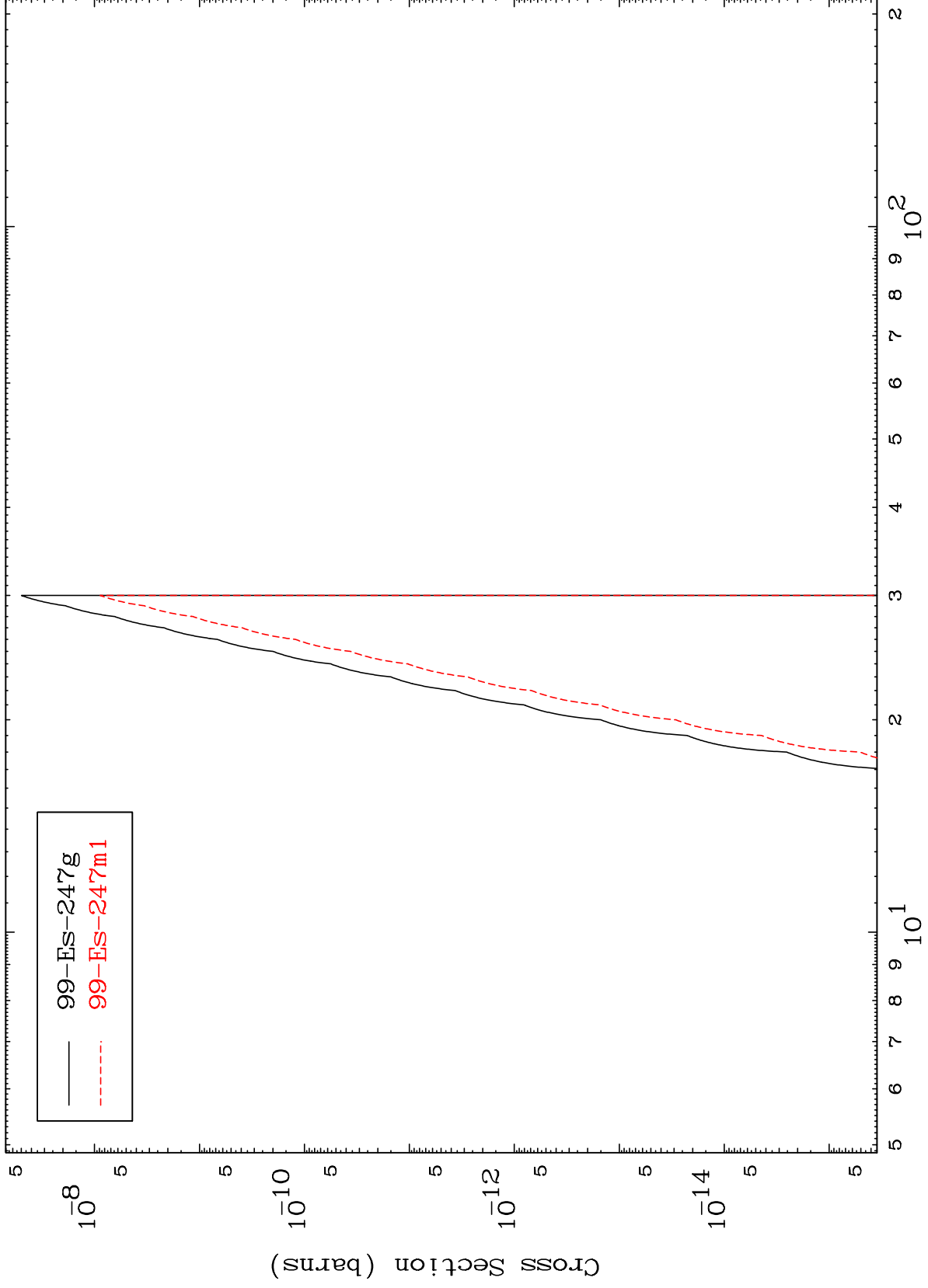
101-Md-250

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(n,n') He-3

101-Md-250

Radionuclide Production Cross Section



29

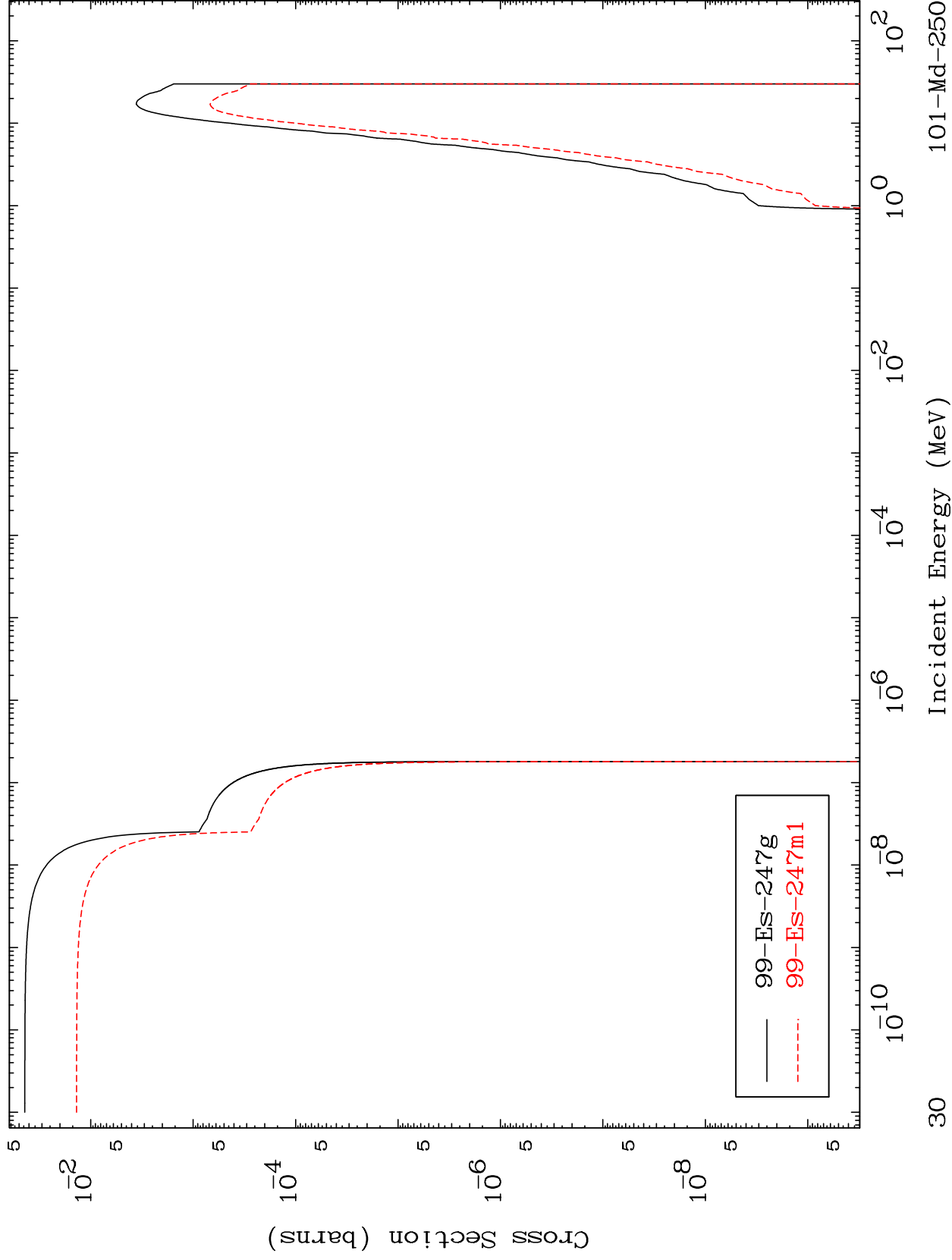
Incident Energy (MeV)

101-Md-250

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101-Md-250

(n,  $\alpha$ )  
Radionuclide Production Cross Section



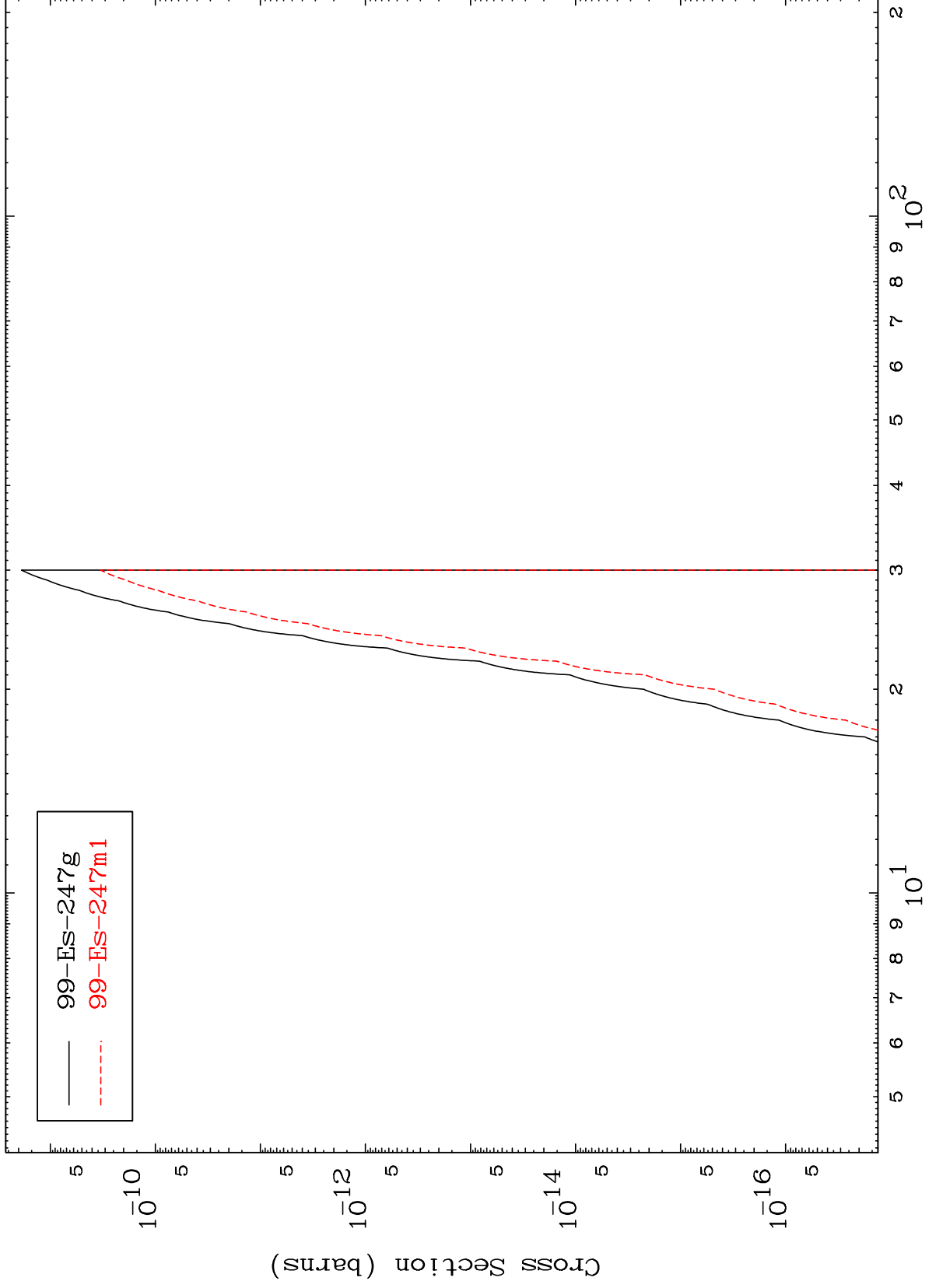
30

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(n,p) t

101-Md-250

Radionuclide Production Cross Section



31

Incident Energy (MeV)

101-Md-250