

Program EVALPLOT
(Version 2021-1)

by

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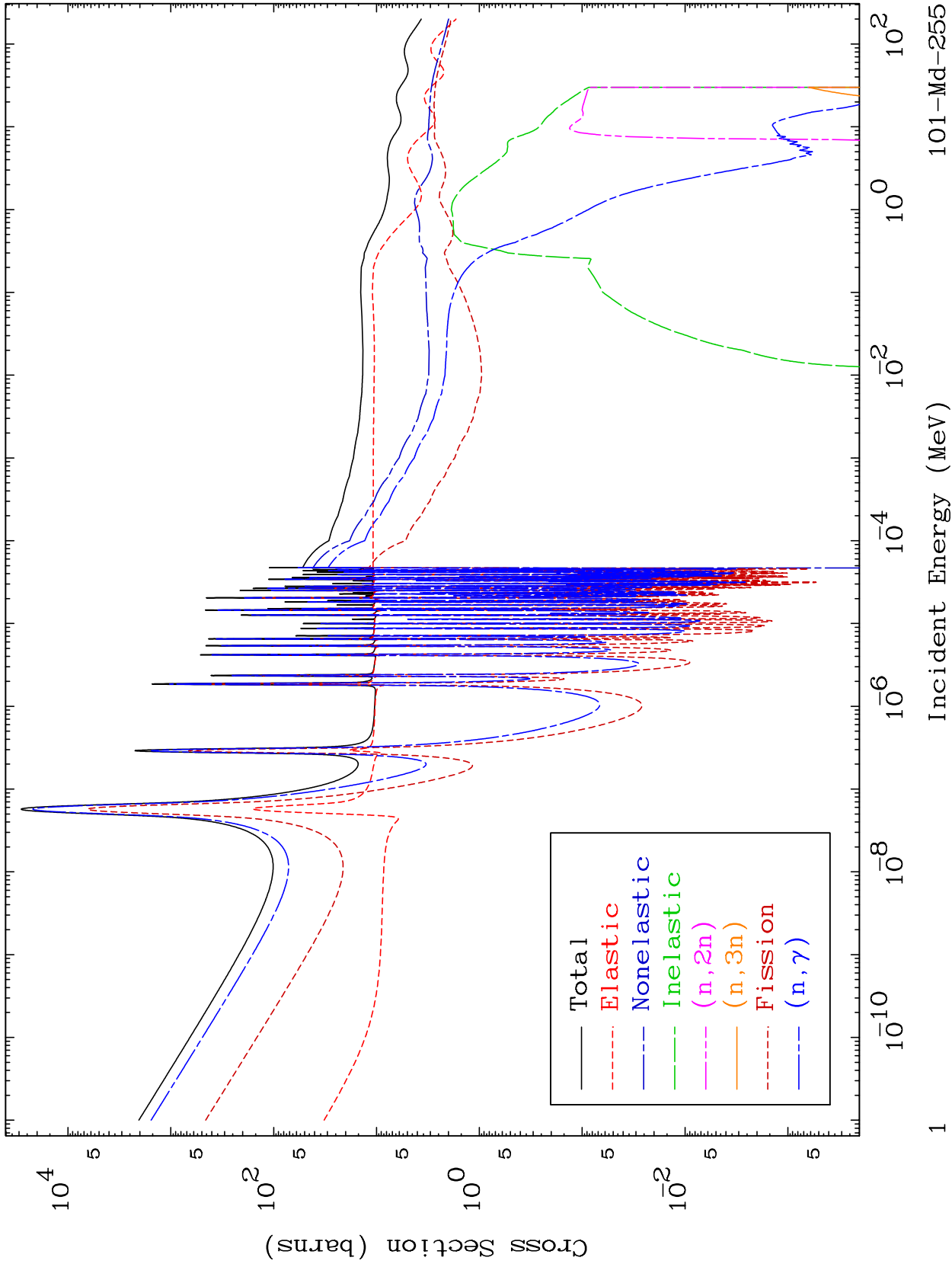
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9956

Neutron Major
293 Kelvin Cross Sections

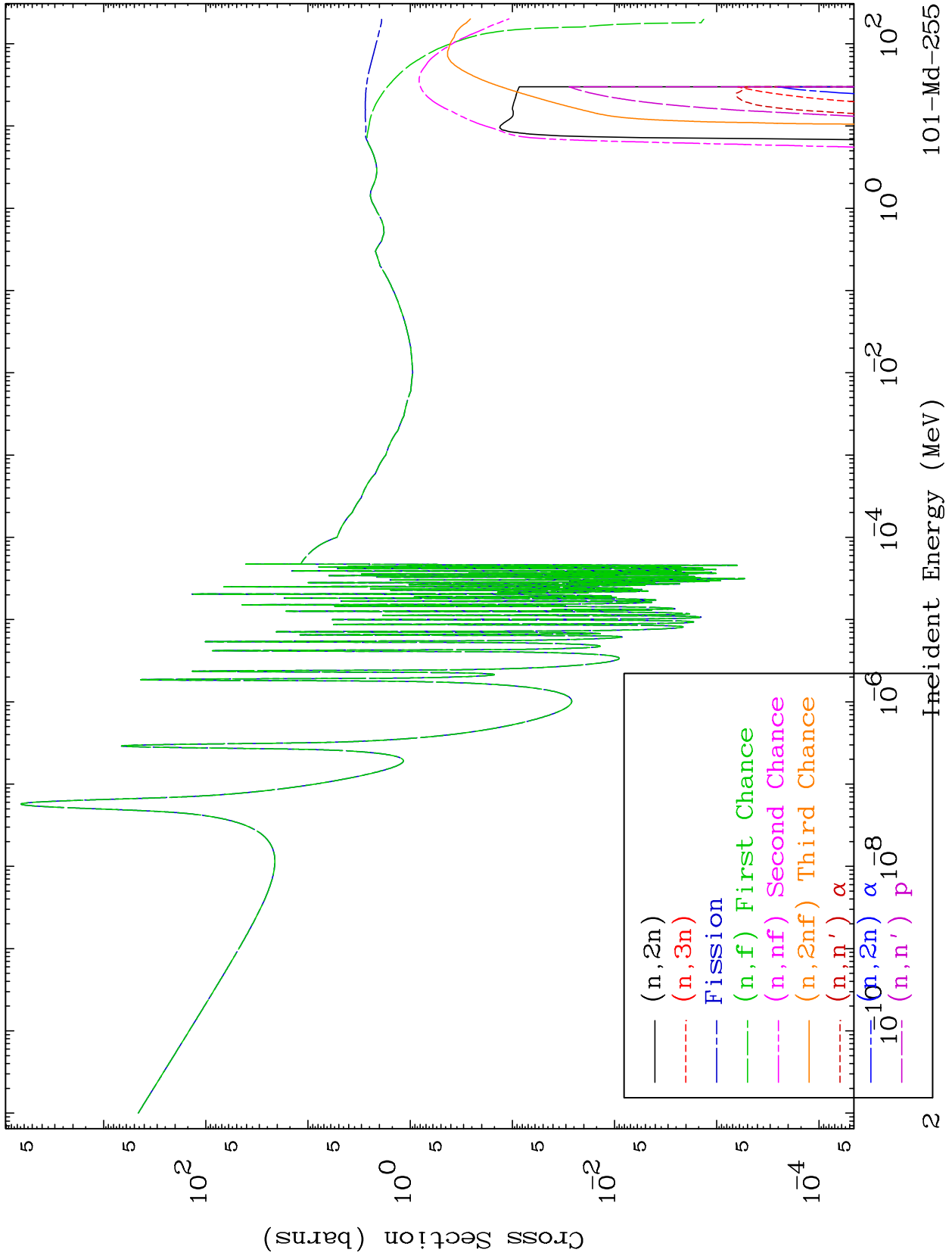
101-Md-255

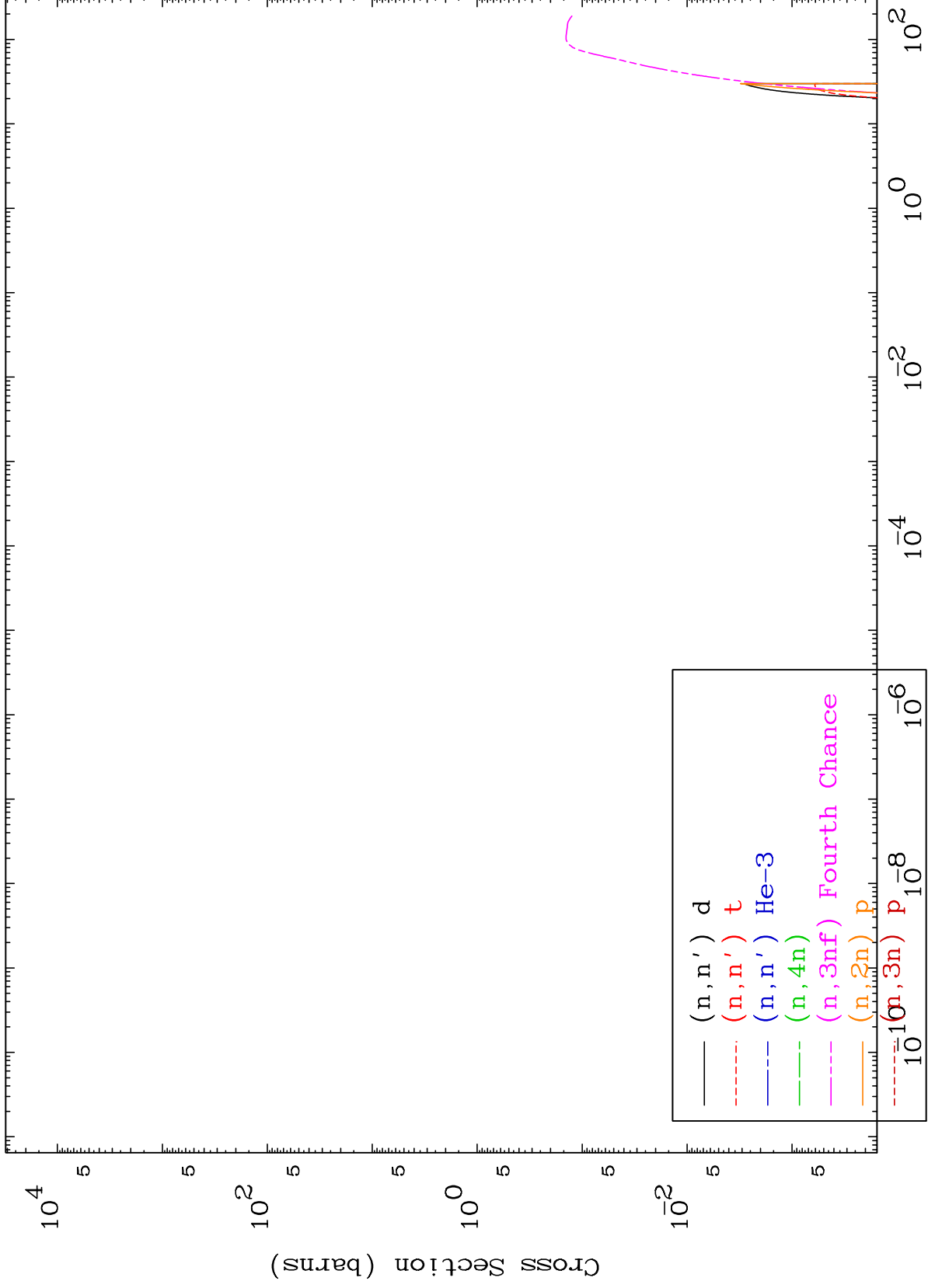


MAT 9956

Neutron Absorption
293 Kelvin Cross Sections

101-Md-255

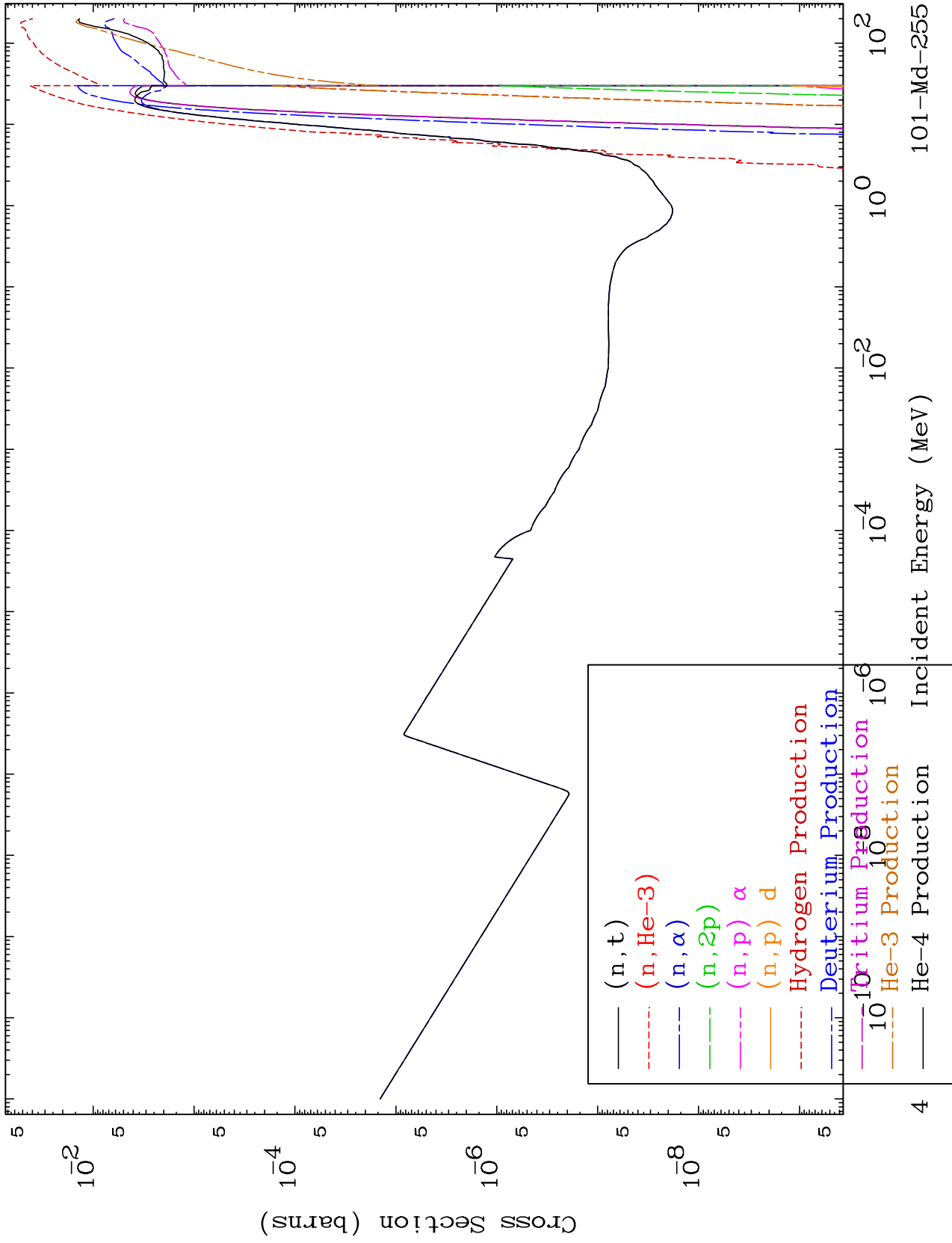




MAT 9956

Neutron Absorption
293 Kelvin Cross Sections

101-Md-255



101-Md-255

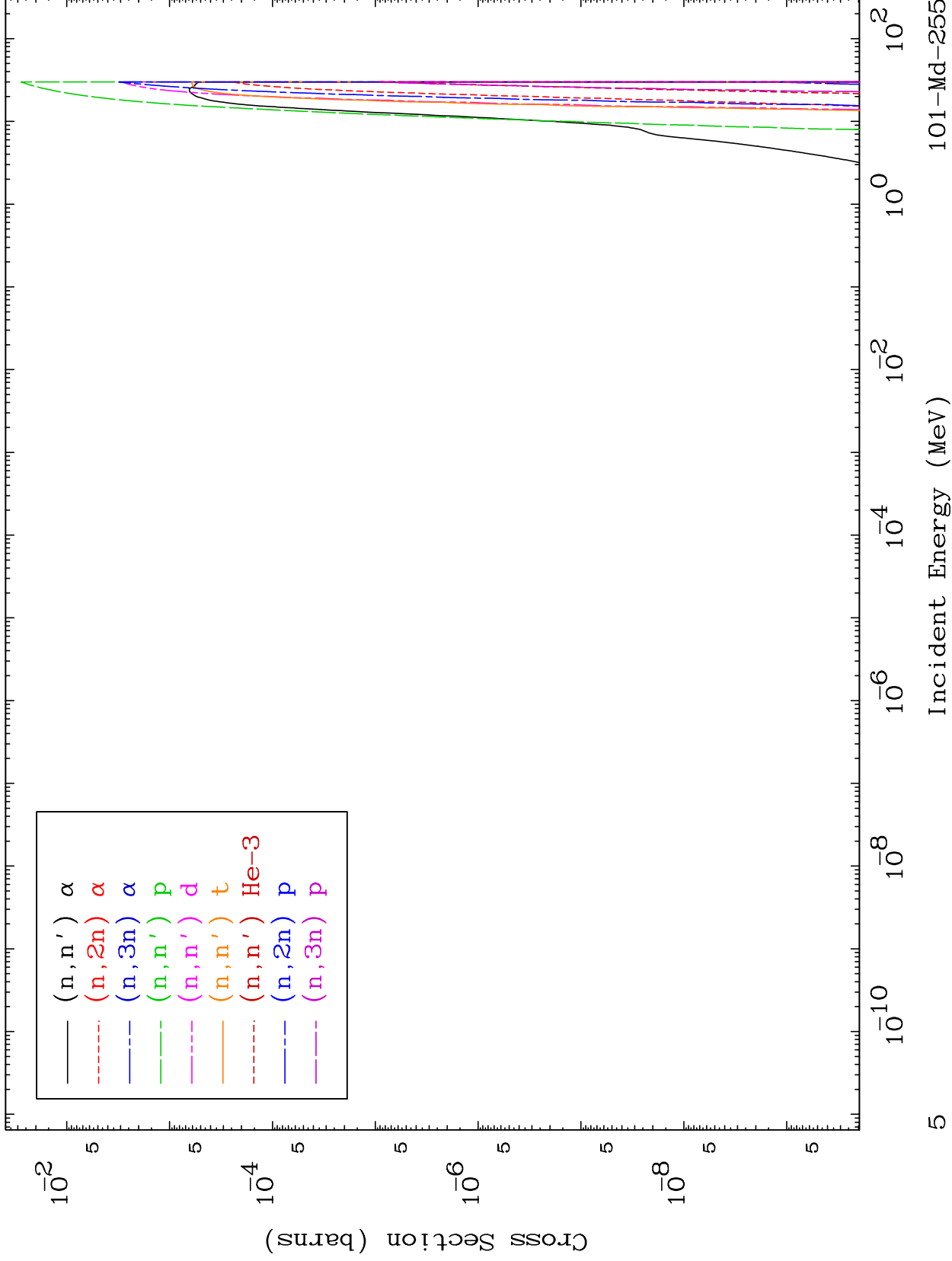
Incident Energy (MeV)

4

MAT 9956

Charged Particle
293 Kelvin Cross Sections

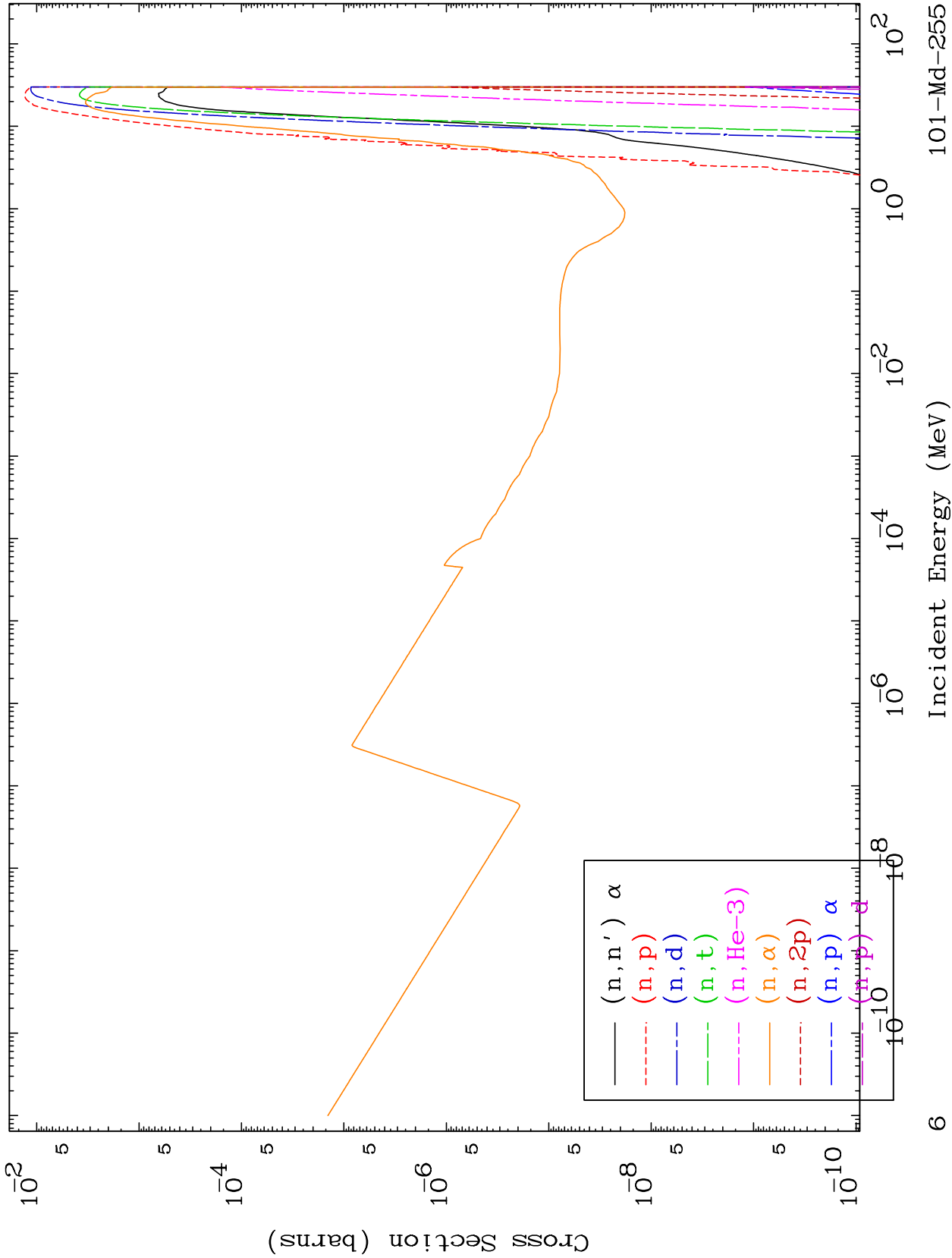
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MAT 9956

Charged Particle
293 Kelvin Cross Sections

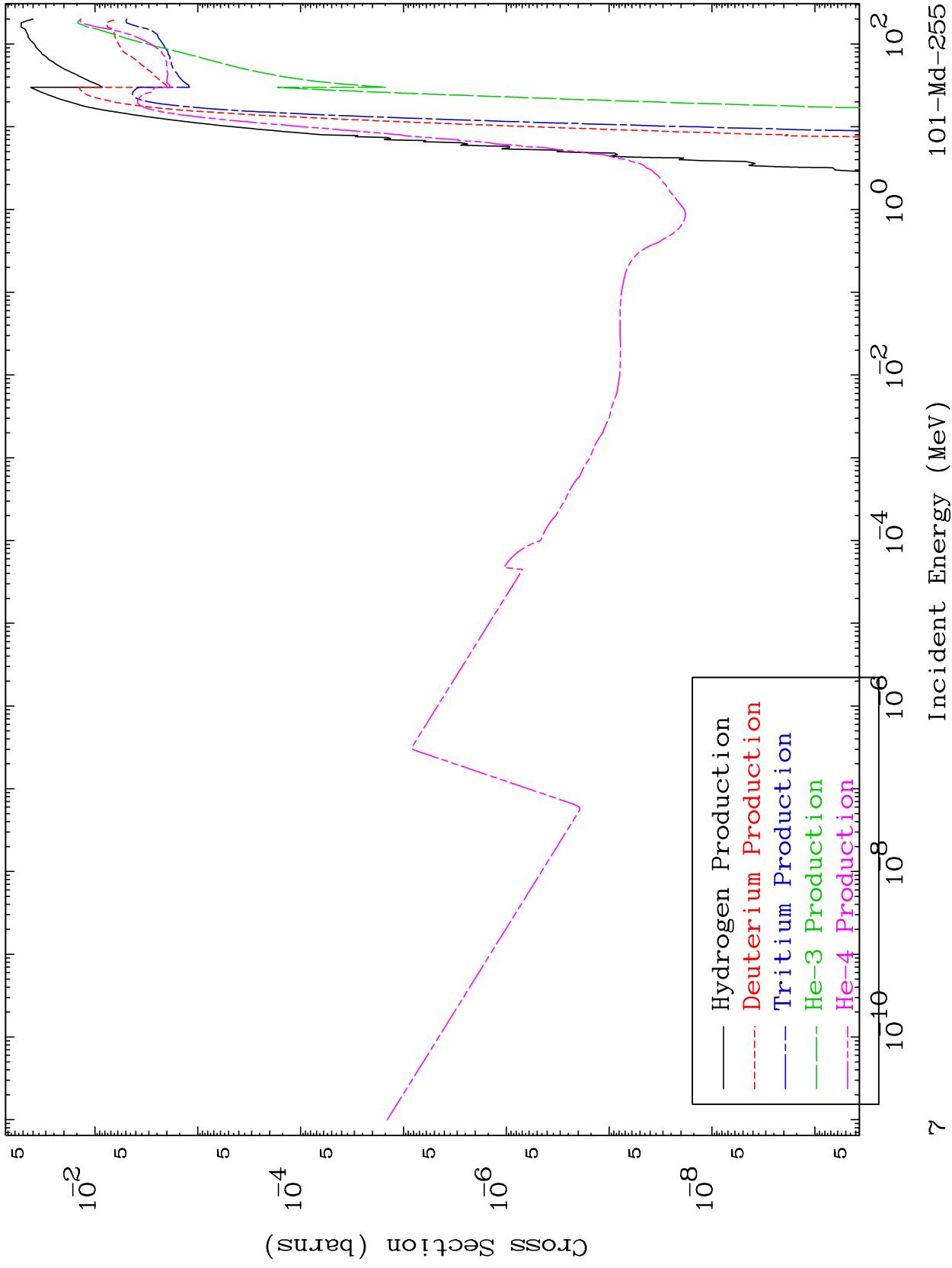
101-Md-255



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Particle Production
293 Kelvin Cross Sections

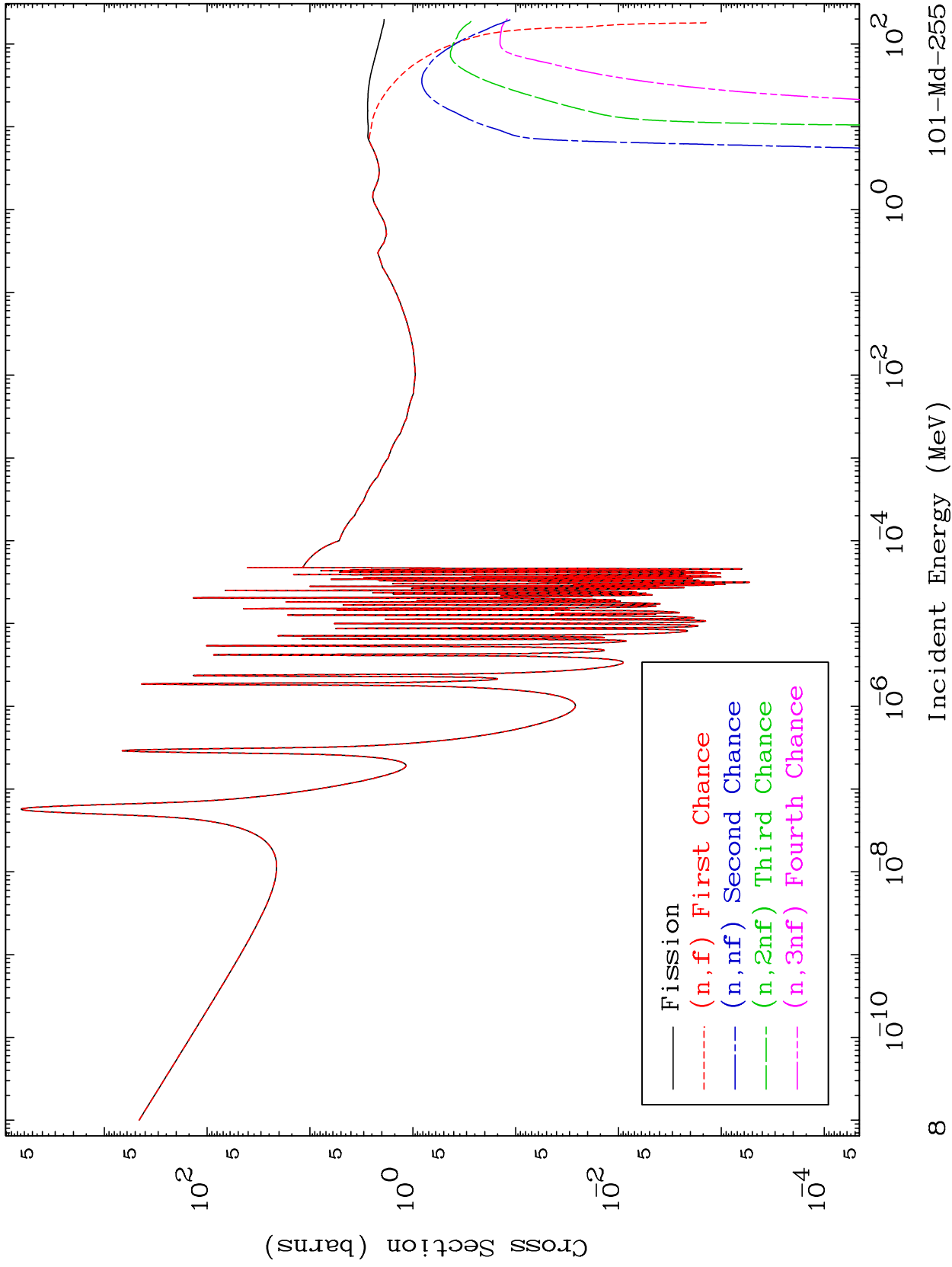
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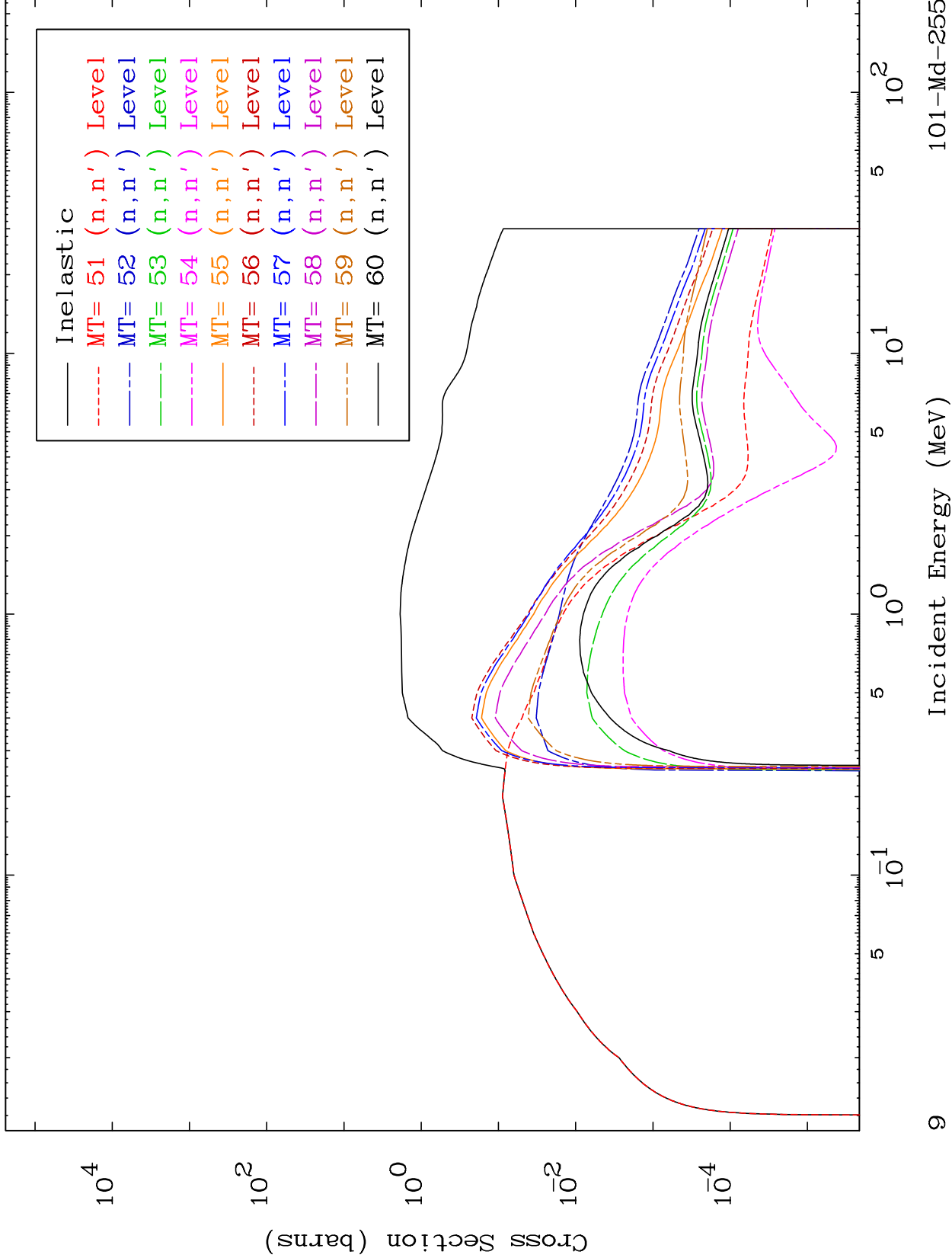


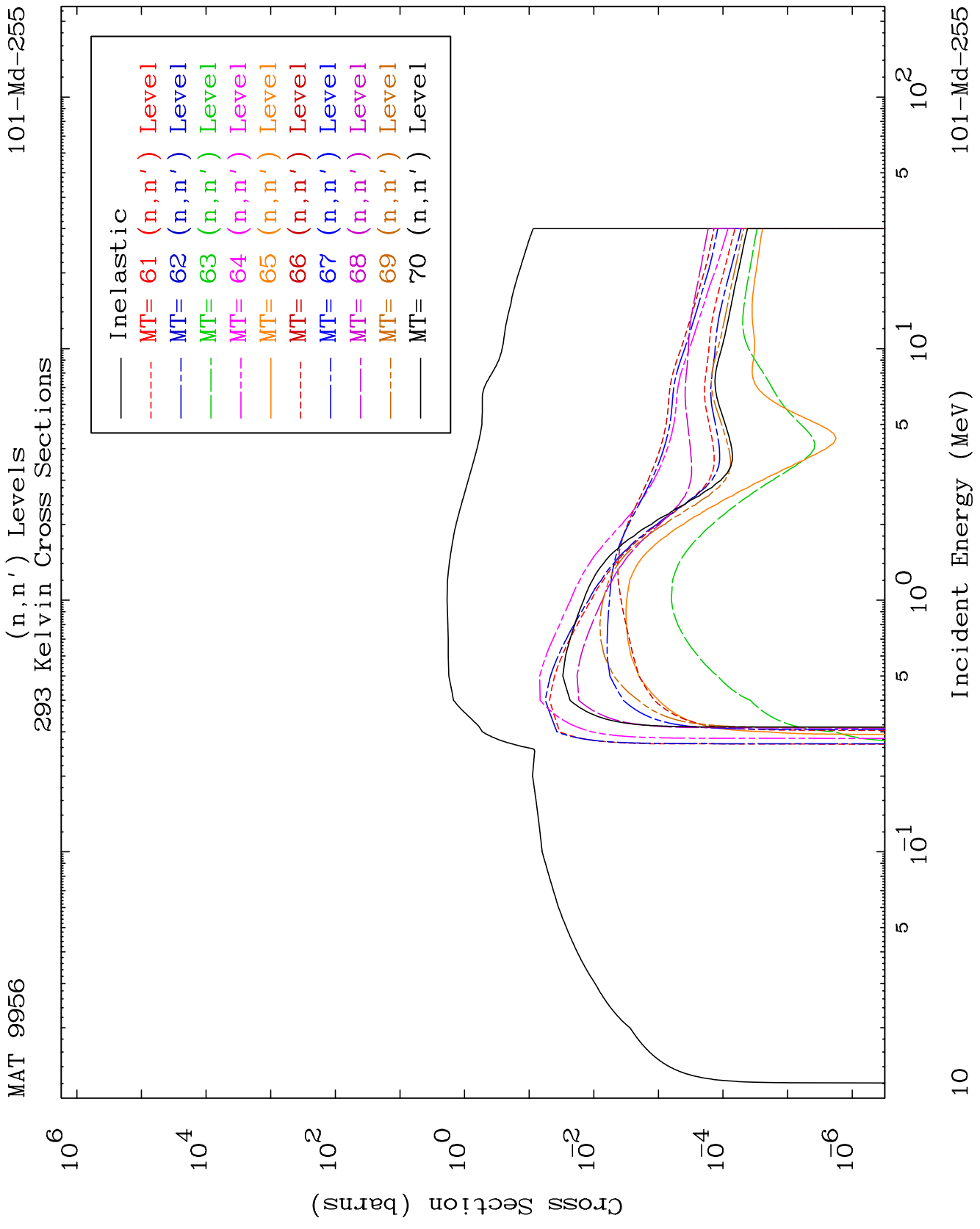
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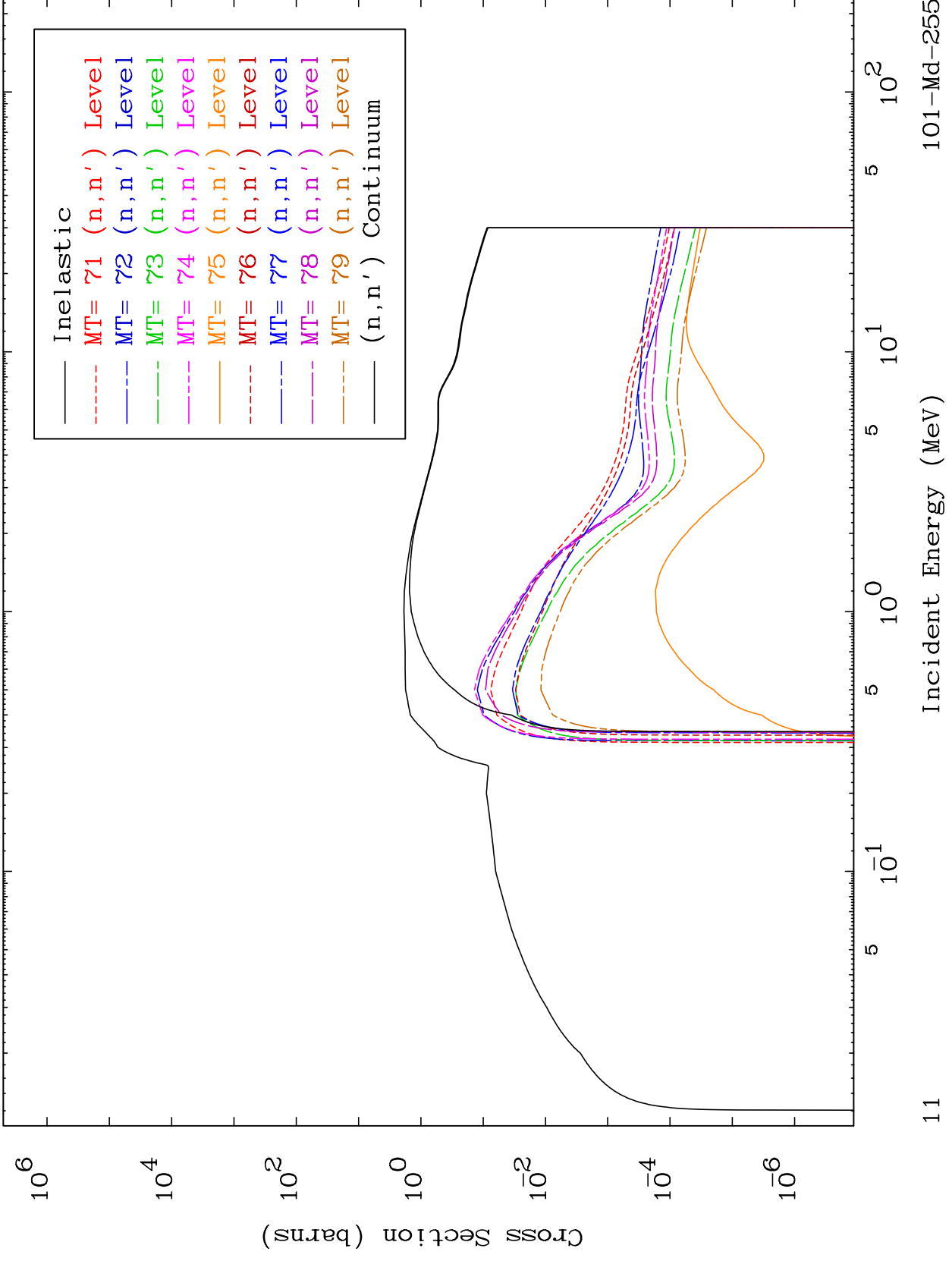
Fission
293 Kelvin Cross Sections

101-Md-255





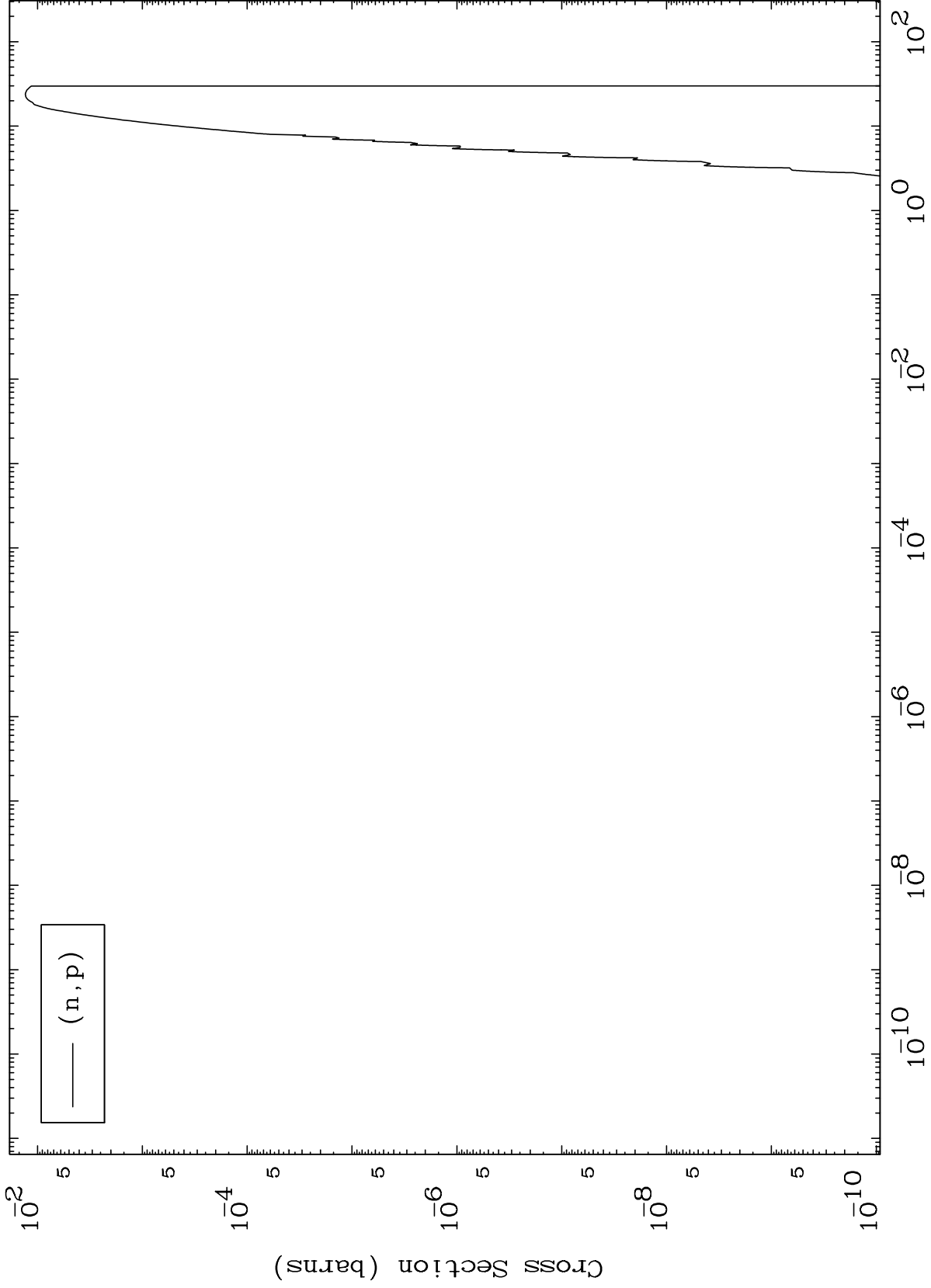




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(n,p) Levels
293 Kelvin Cross Sections

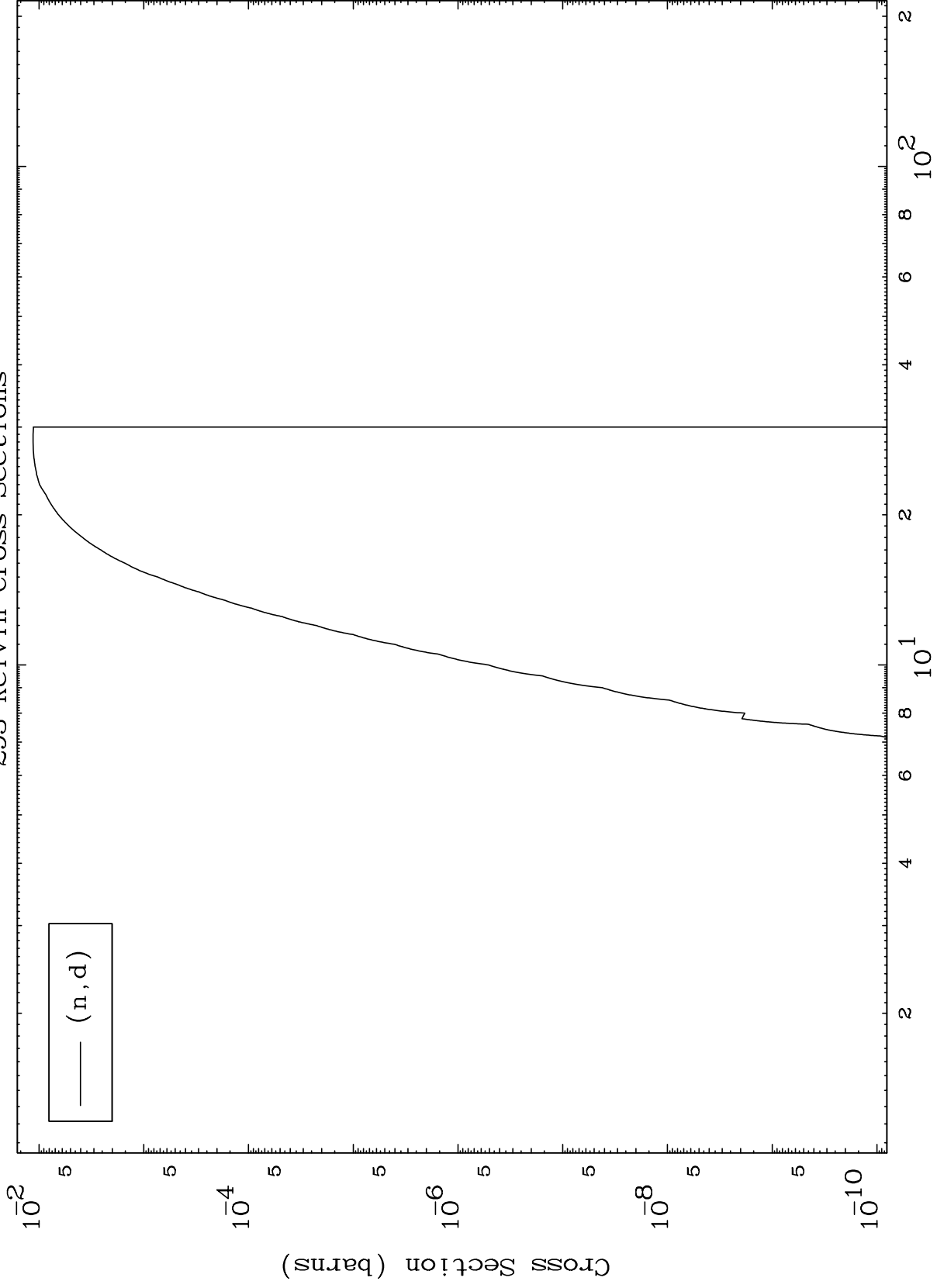
101-Md-255



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(n,d) Levels
293 Kelvin Cross Sections

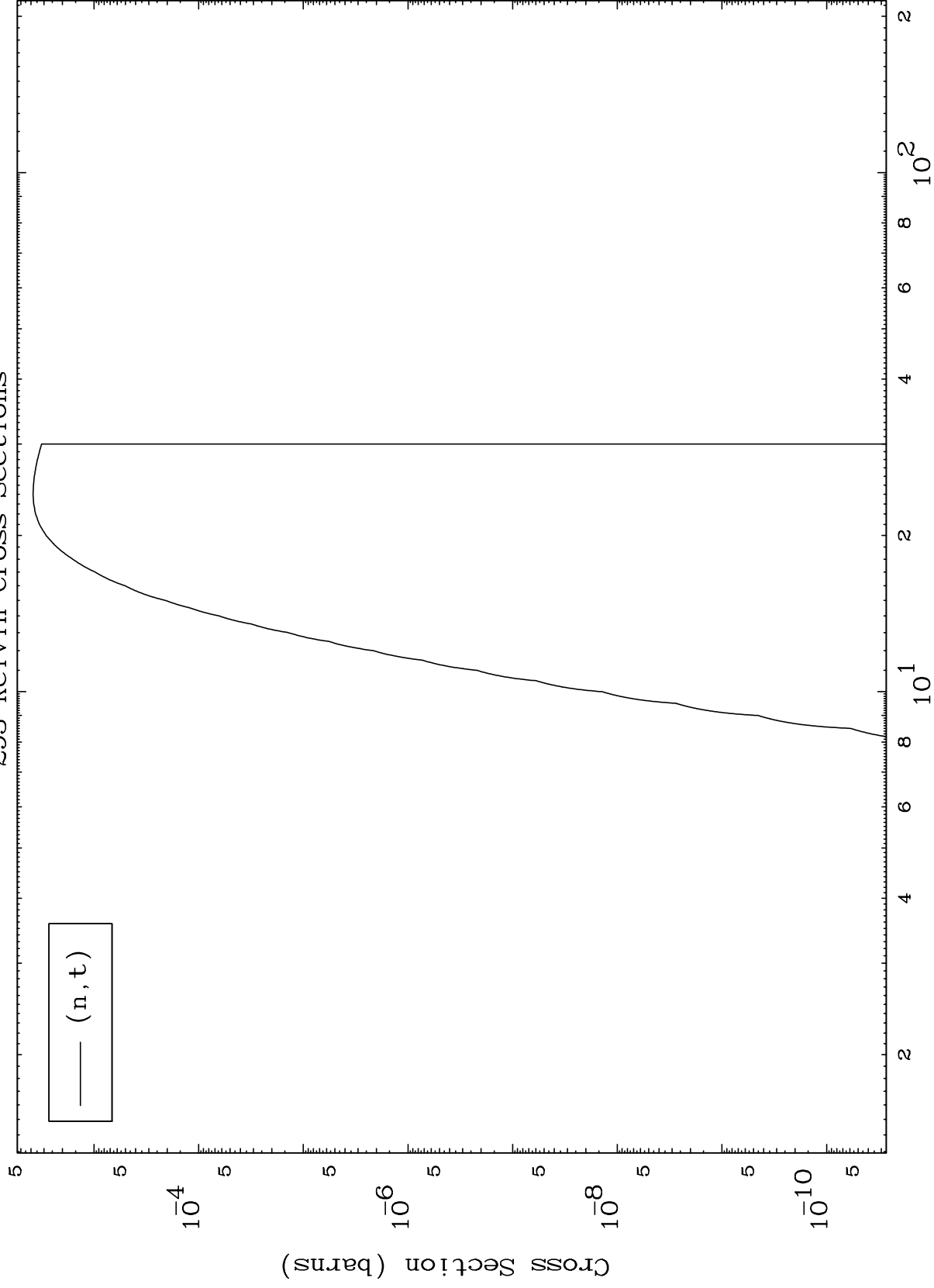
101-Md-255



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(n,t) Levels
293 Kelvin Cross Sections

101-Md-255



14

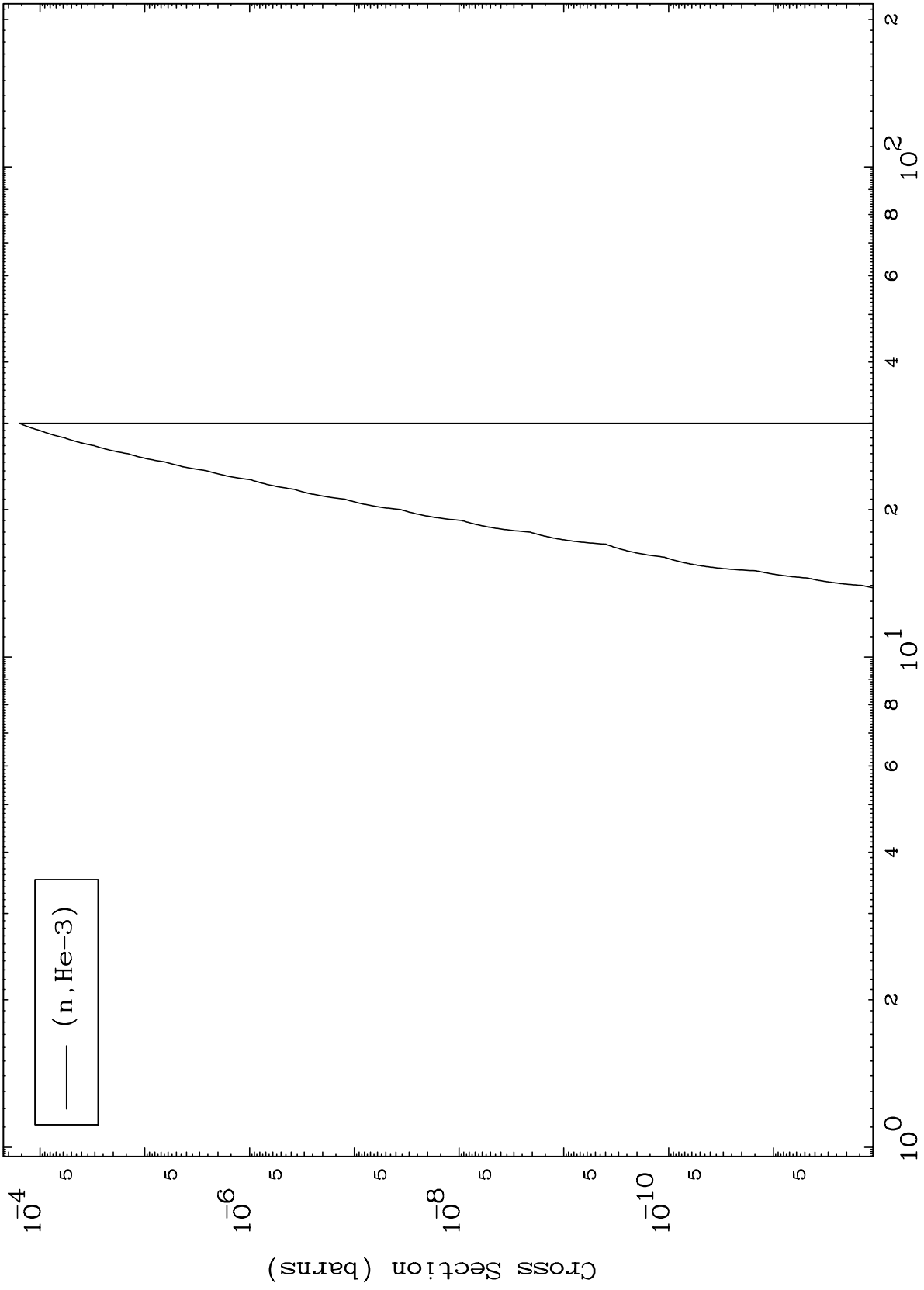
Incident Energy (MeV)

101-Md-255

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(n,He3) Levels
293 Kelvin Cross Sections

101-Md-255



15

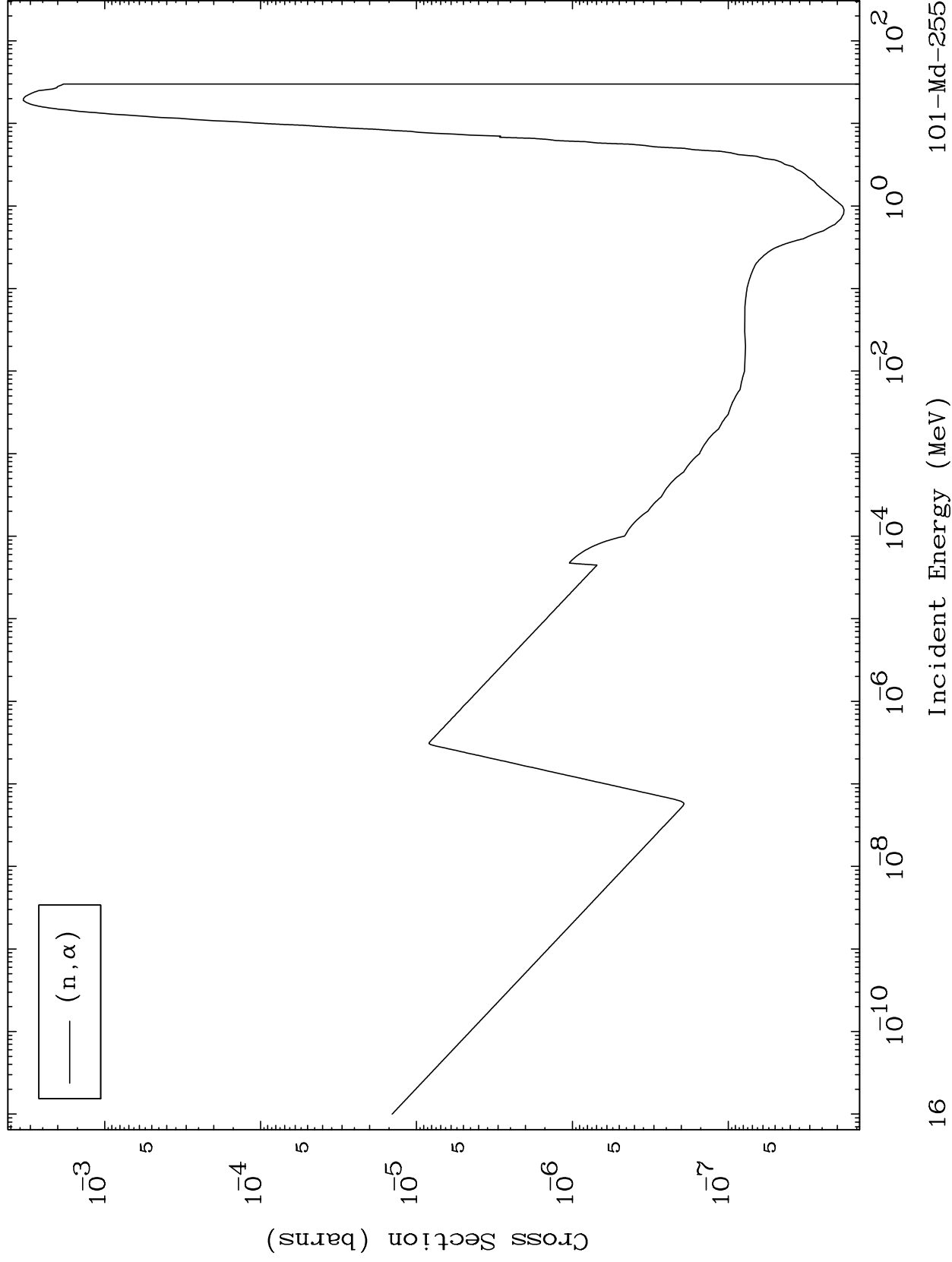
Incident Energy (MeV)

101-Md-255

MAT 9956

(n, α) Levels
293 Kelvin Cross Sections

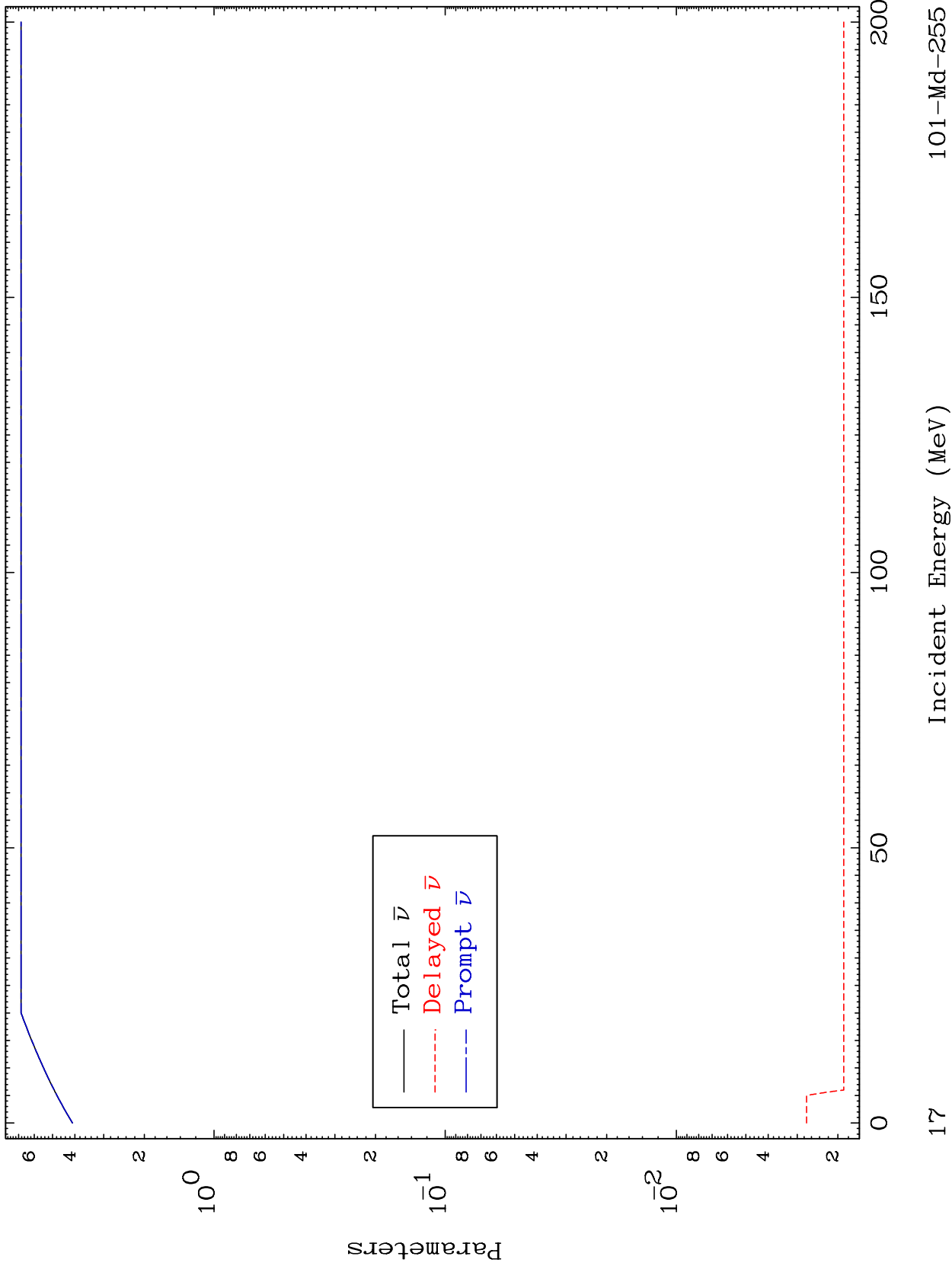
101-Md-255



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Energy Release
Parameters

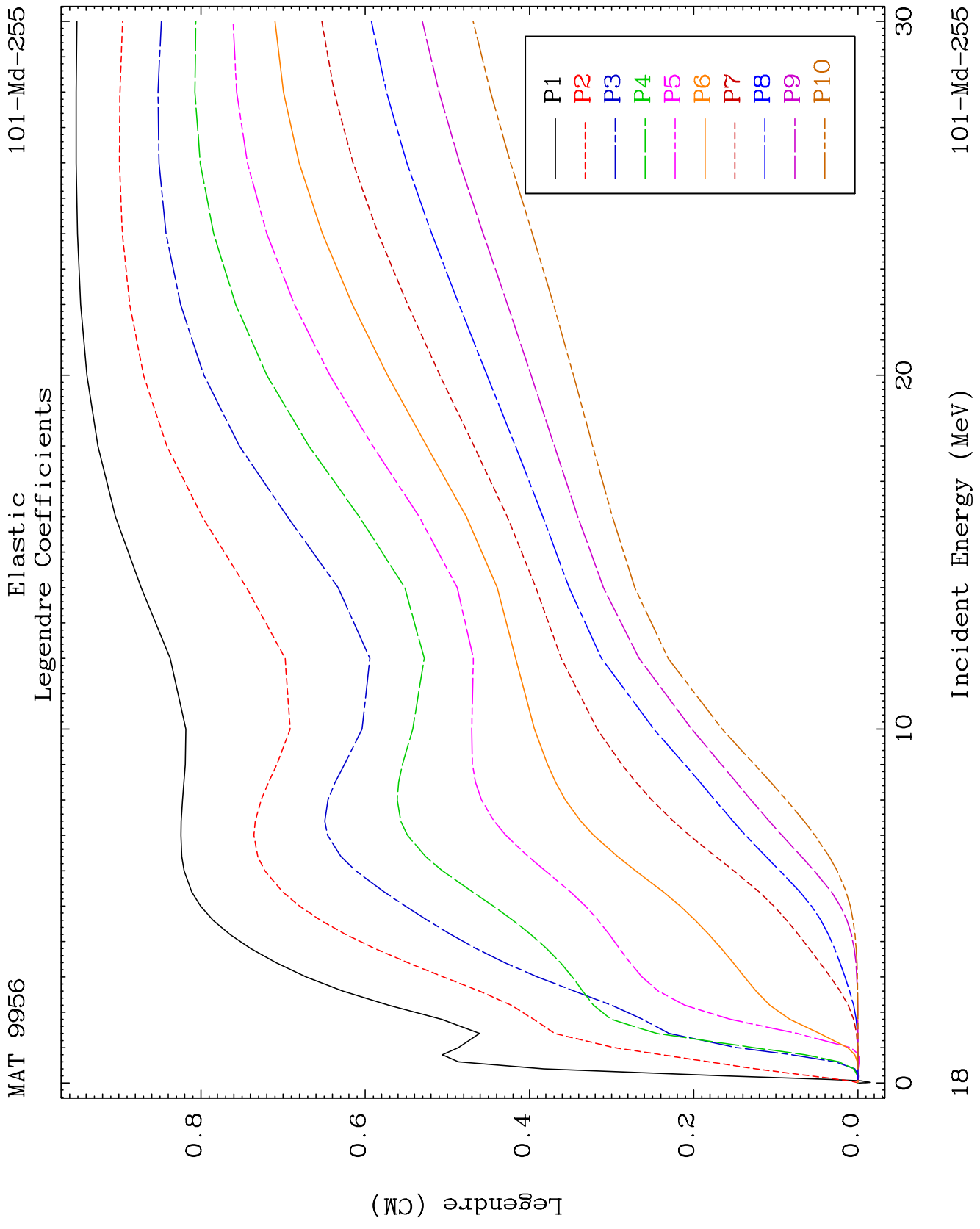
101-Md-255



17

Incident Energy (MeV)

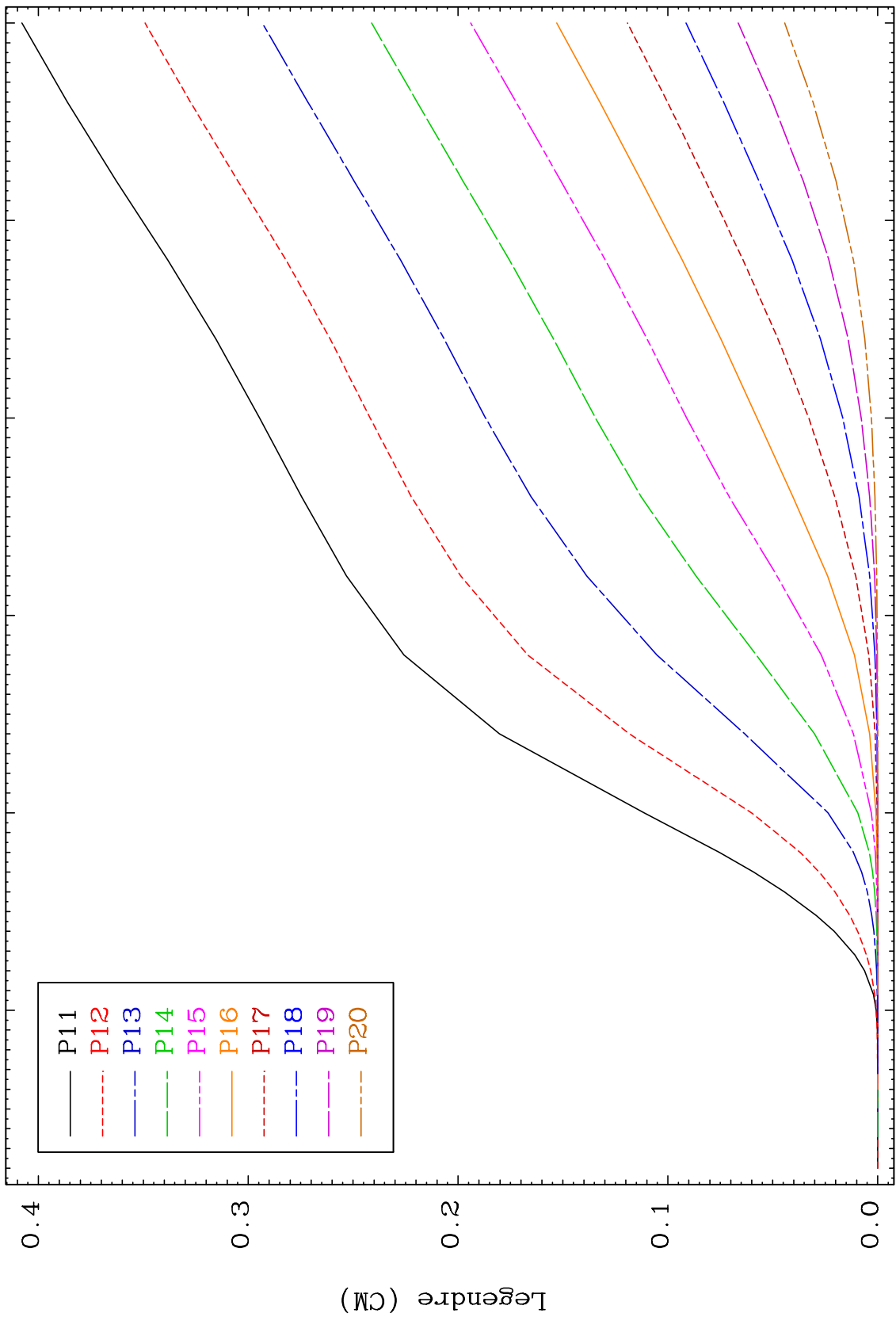
101-Md-255



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Elastic Legendre Coefficients

101-Md-255



19

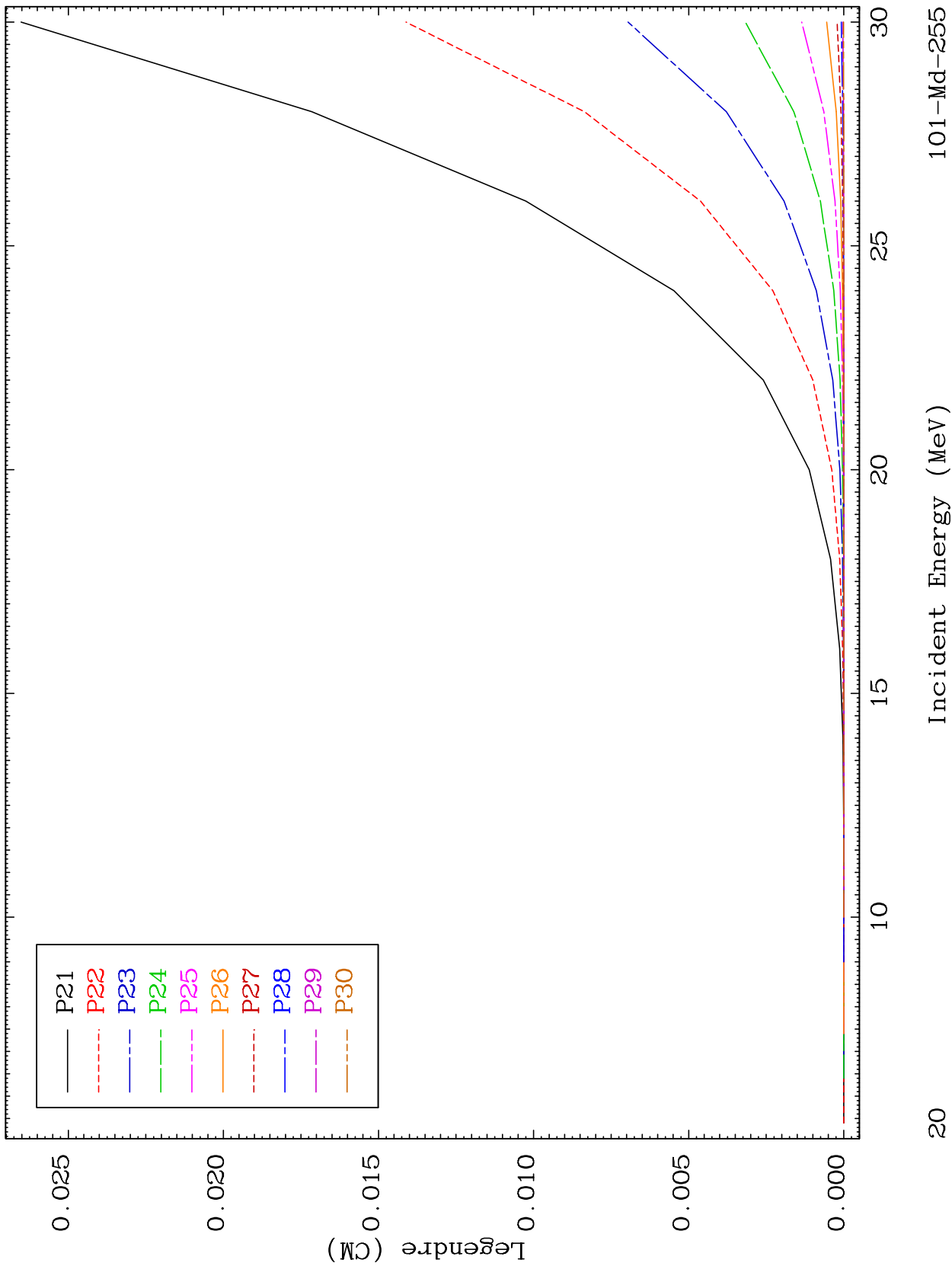
Incident Energy (MeV)

101-Md-255

MAT 9956

Elastic Legendre Coefficients

101-Md-255



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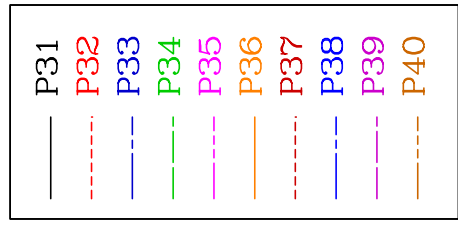
Incident Energy (MeV)

101-Md-255

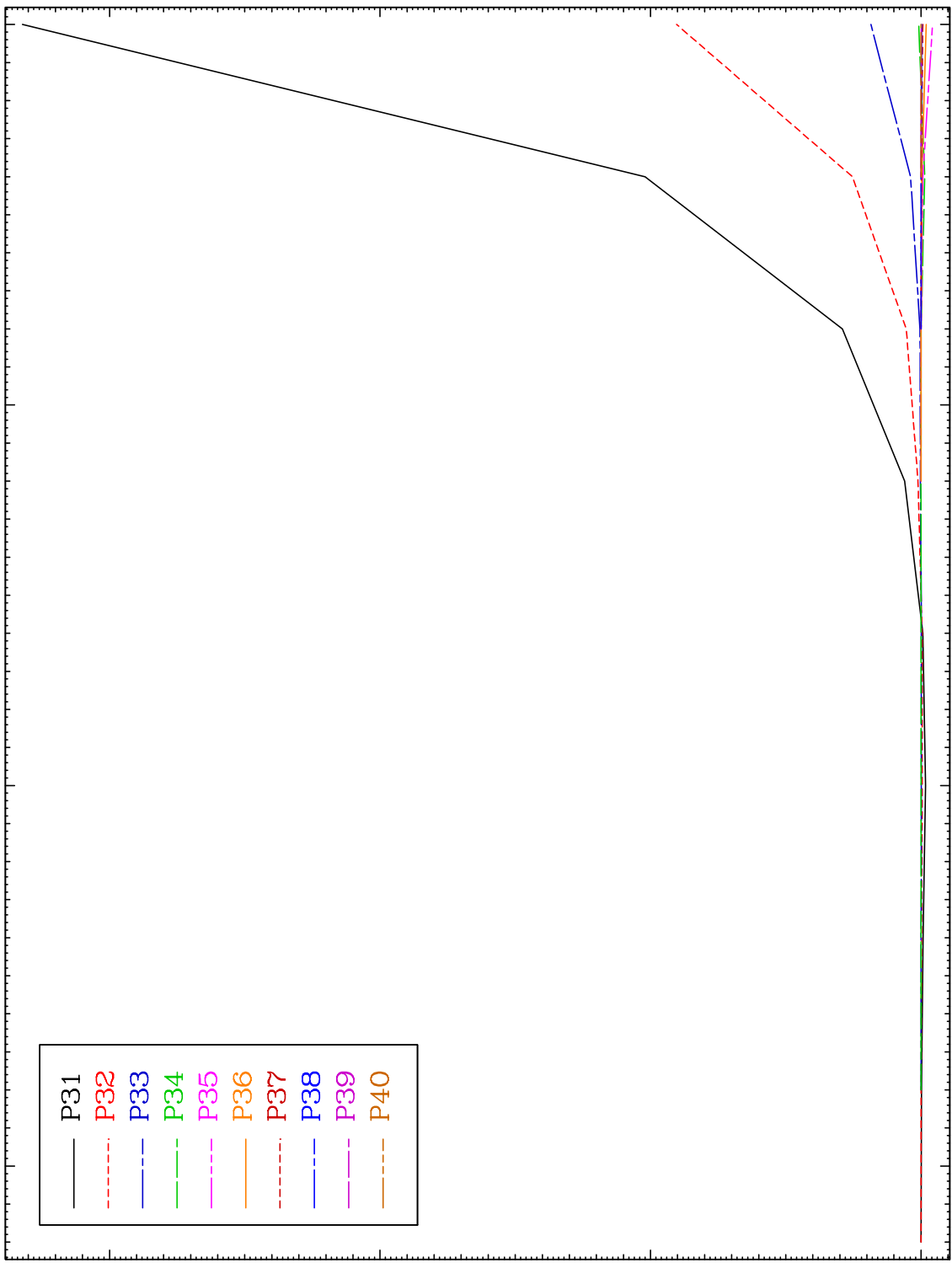
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Elastic Legendre Coefficients

101-Md-255



Legendre (CM)
 $\times 10^{-6}$



21

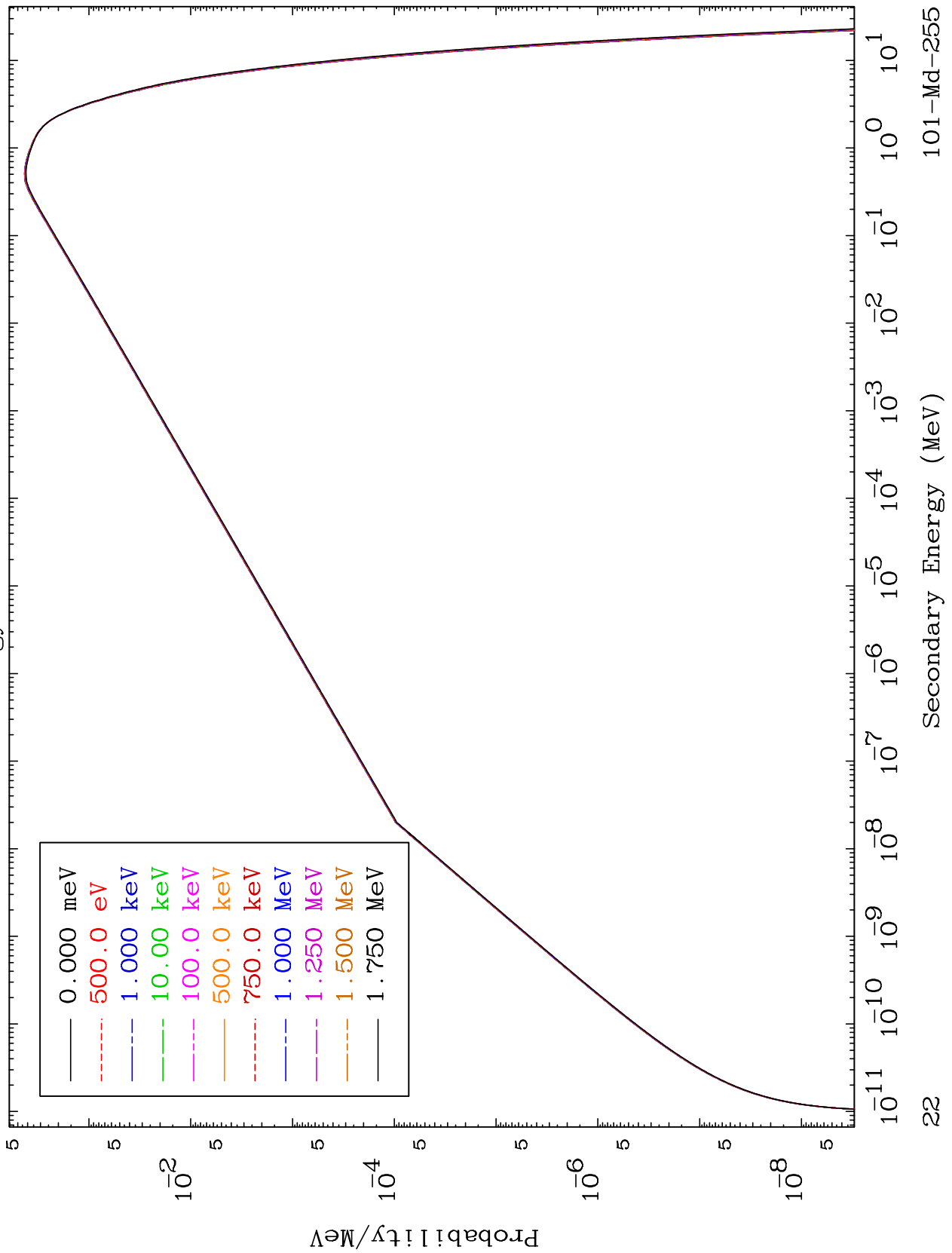
Incident Energy (MeV)

101-Md-255

MAT 9956

Fission Energy Distributions

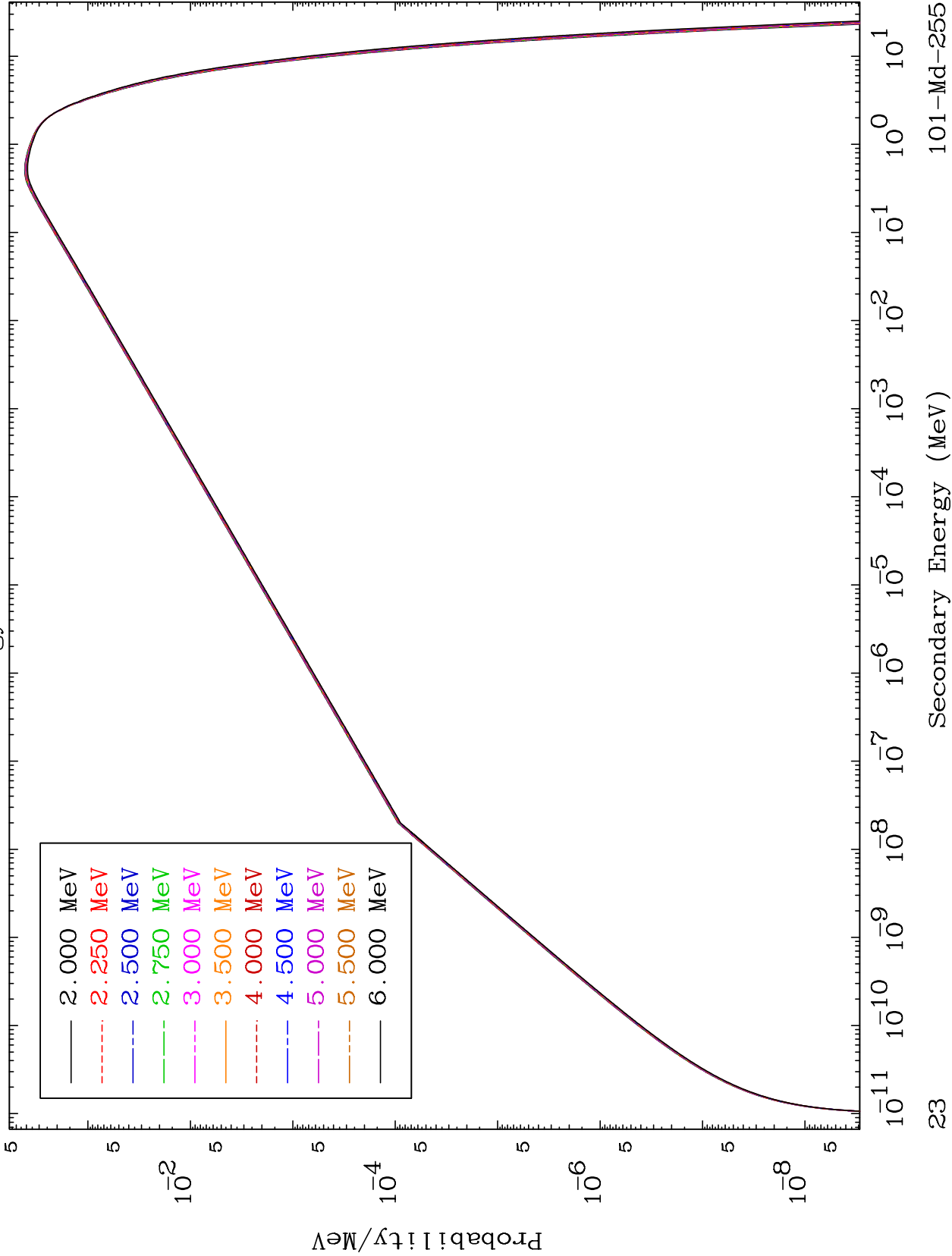
101-Md-255



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Fission Energy Distributions

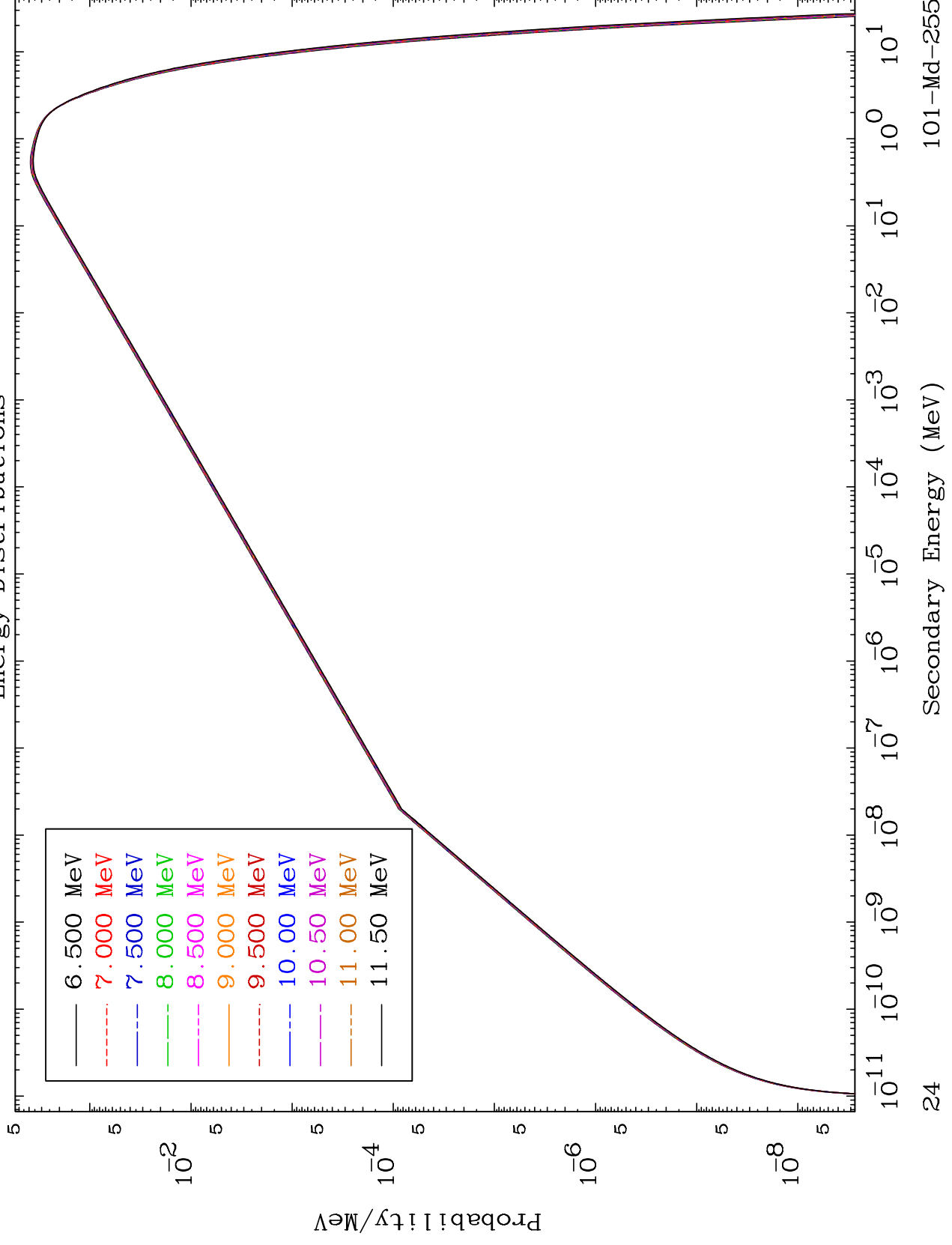
101-Md-255



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Fission Energy Distributions

101-Md-255



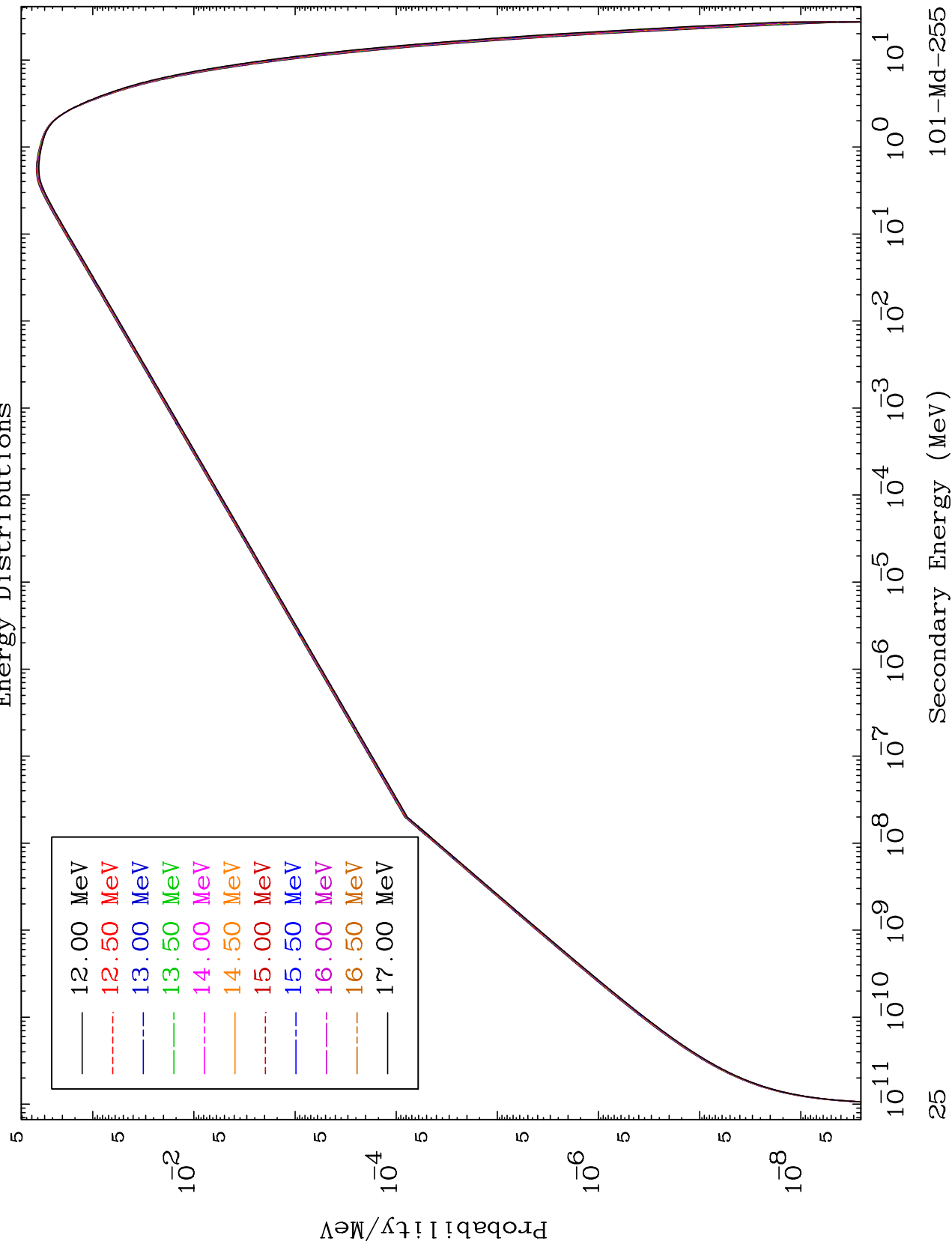
101-Md-255

Secondary Energy (MeV)

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Fission Energy Distributions

101-Md-255

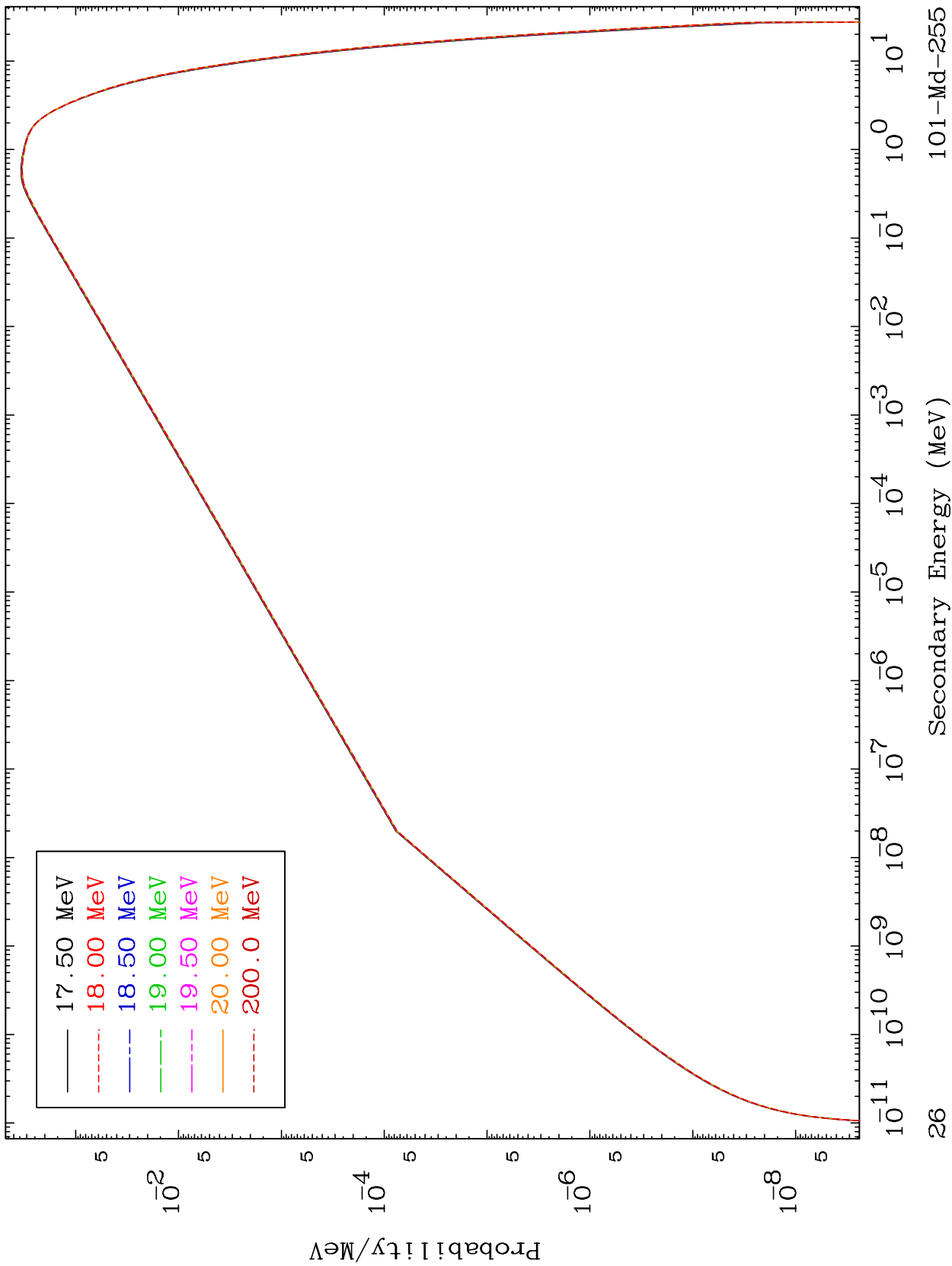


101-Md-255

Secondary Energy (MeV)

25

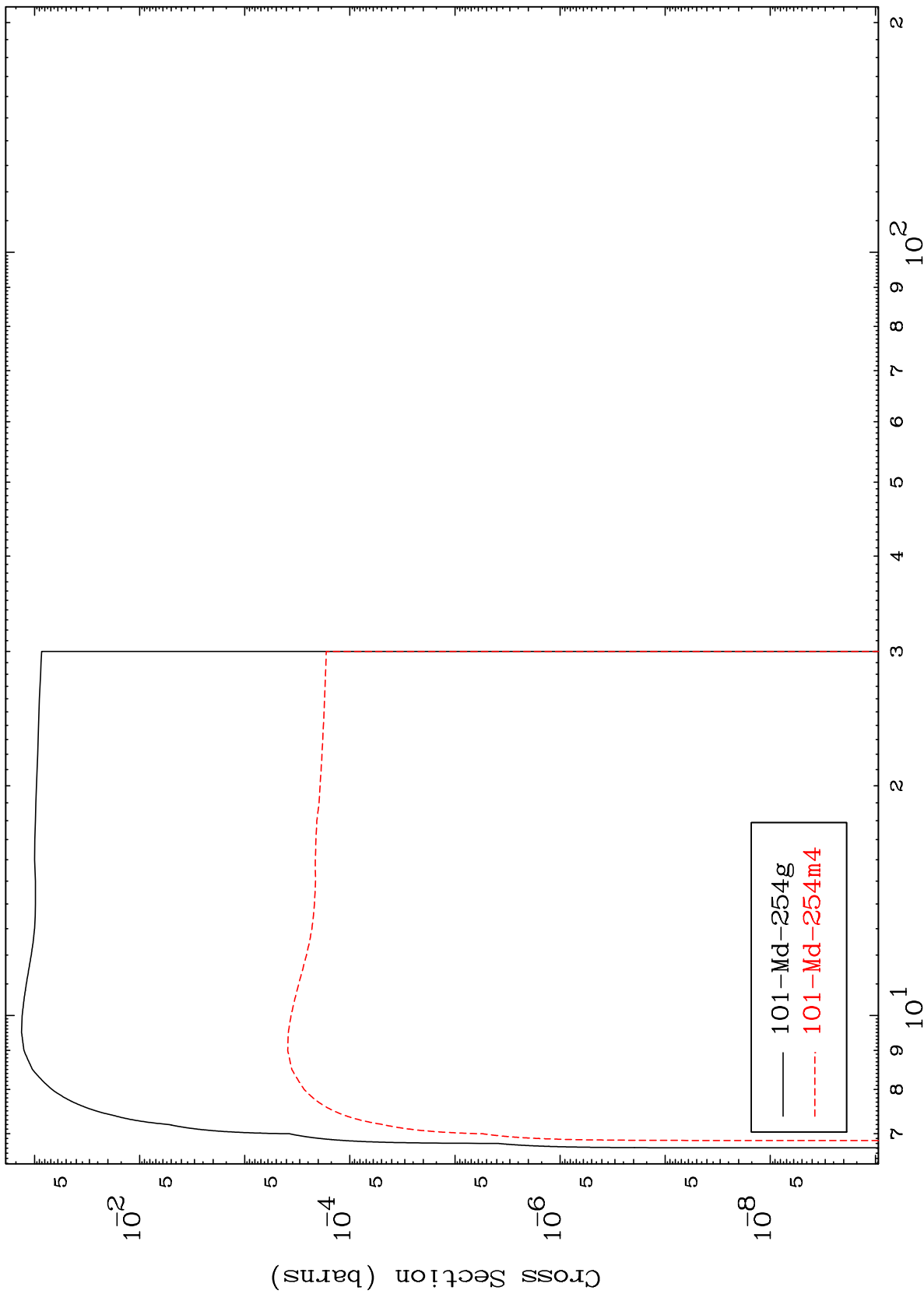
Fission Energy Distributions



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101-Md-255

(n,2n)
Radionuclide Production Cross Section



101-Md-255

Incident Energy (MeV)

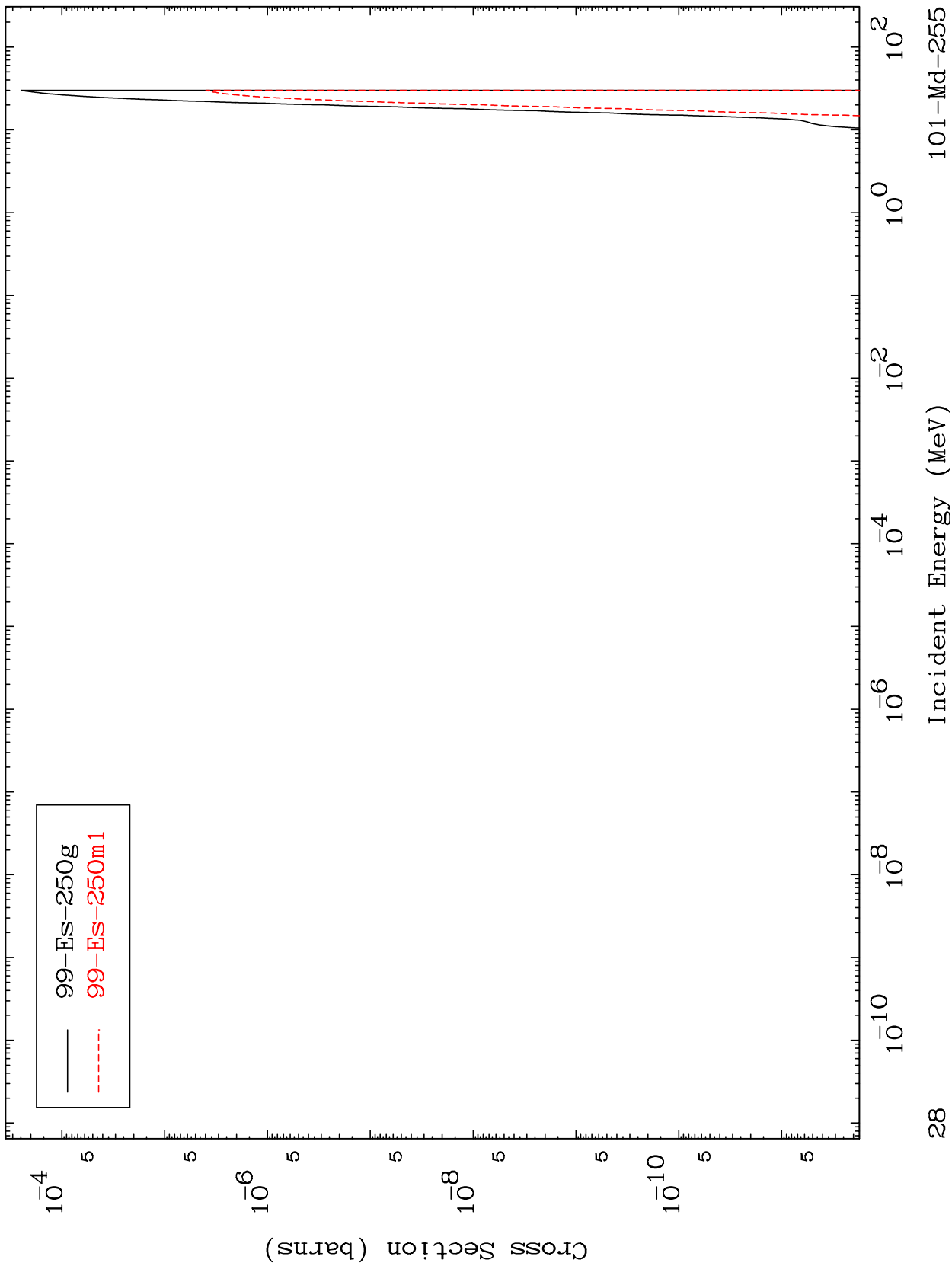
27

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(n,2n) α

101-Md-255

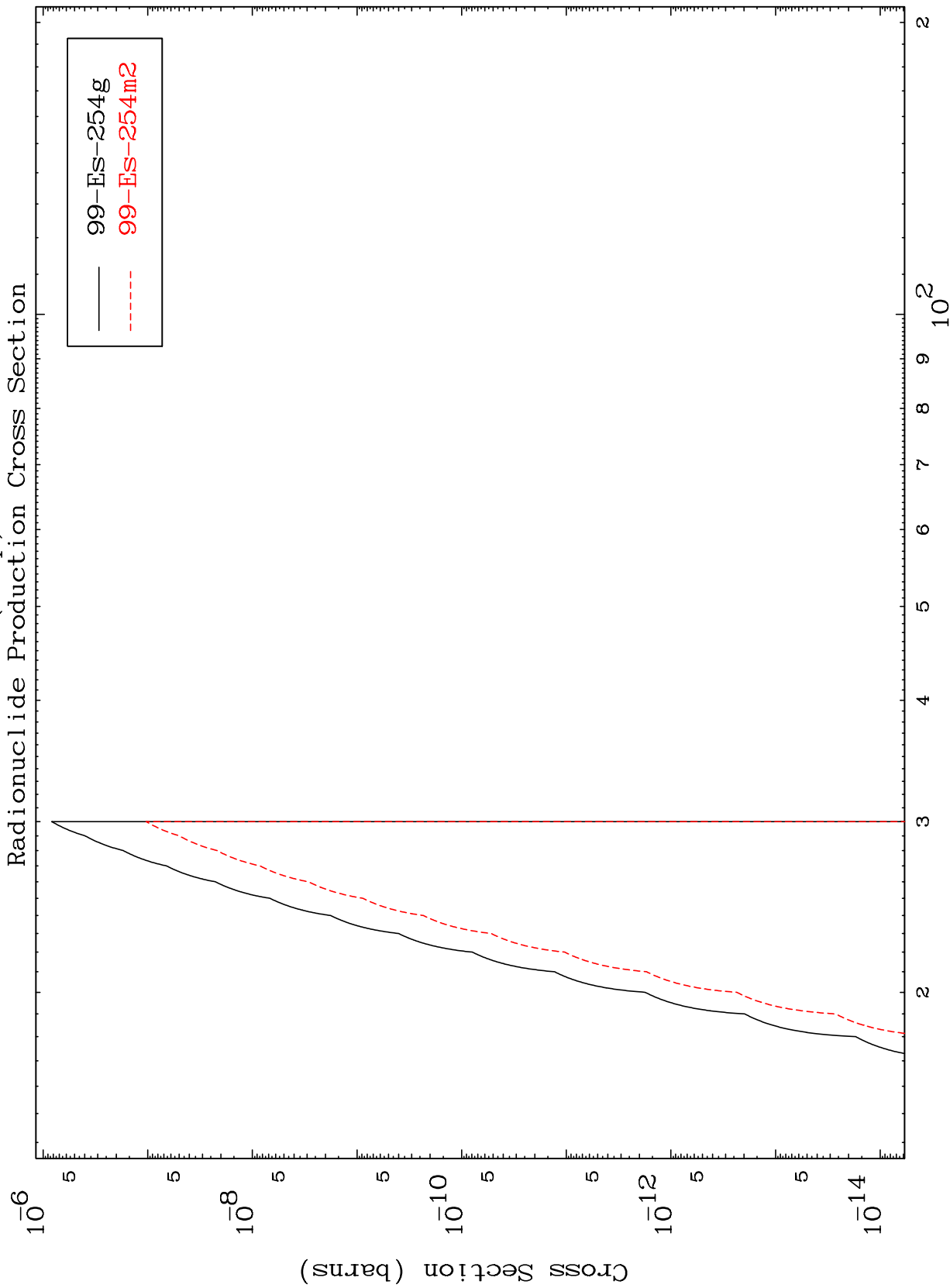
Radionuclide Production Cross Section



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101-Md-255

(n,2p)



29

Incident Energy (MeV)

101-Md-255