

Program EVALPLOT
(Version 2021-1)

by

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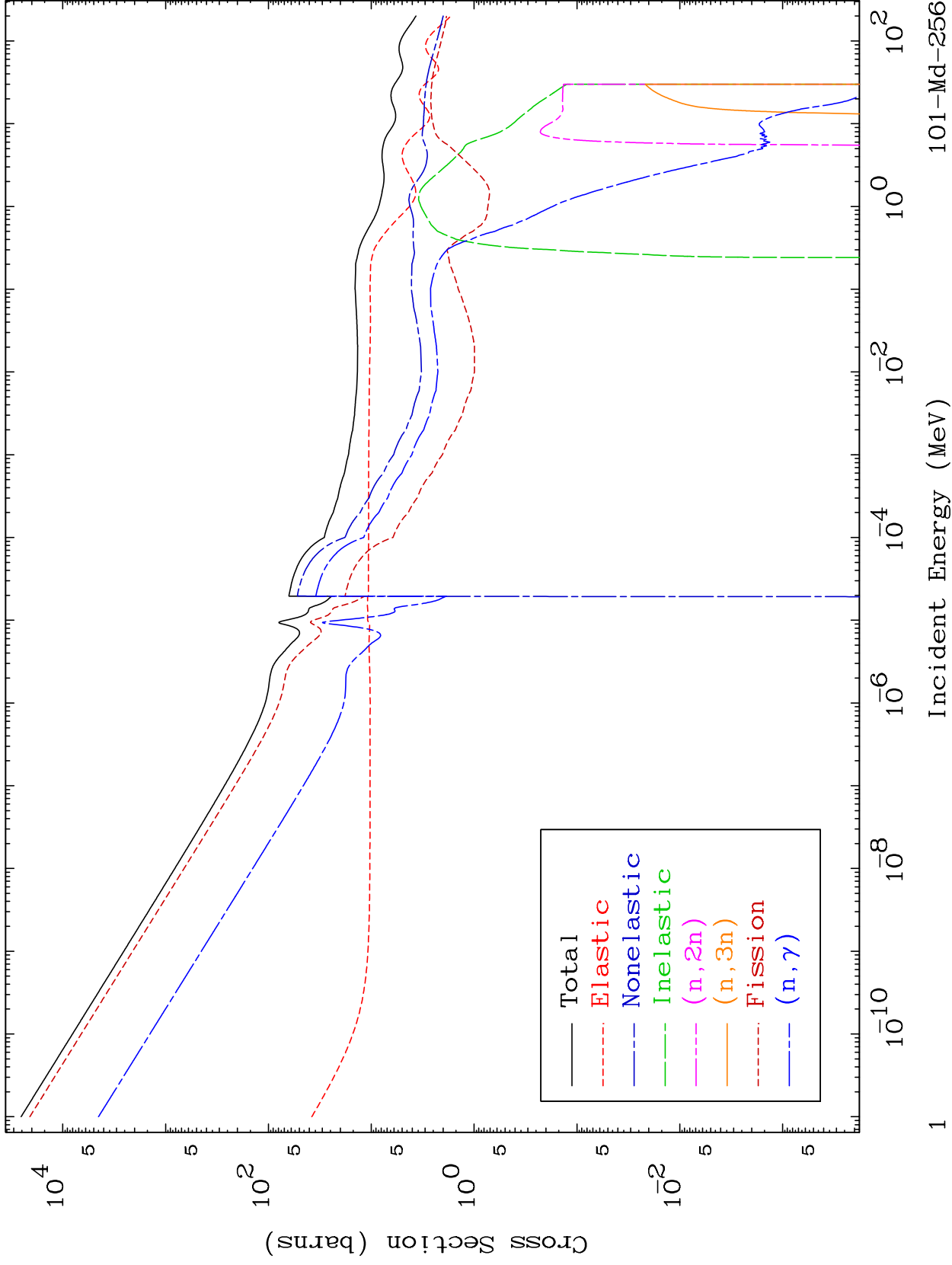
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Press Mouse Button to Start

MAT 9957

Neutron Major
293 Kelvin Cross Sections

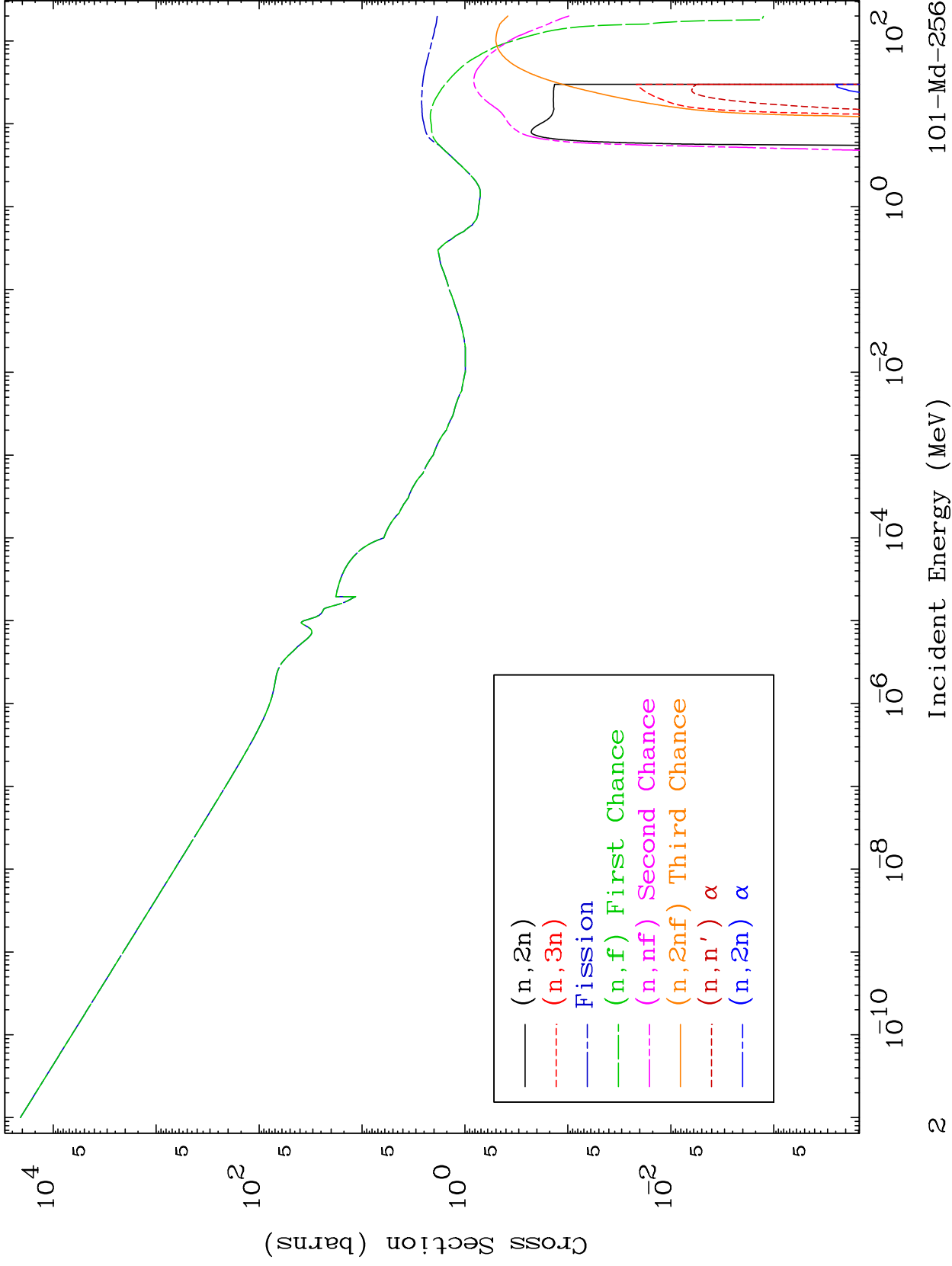
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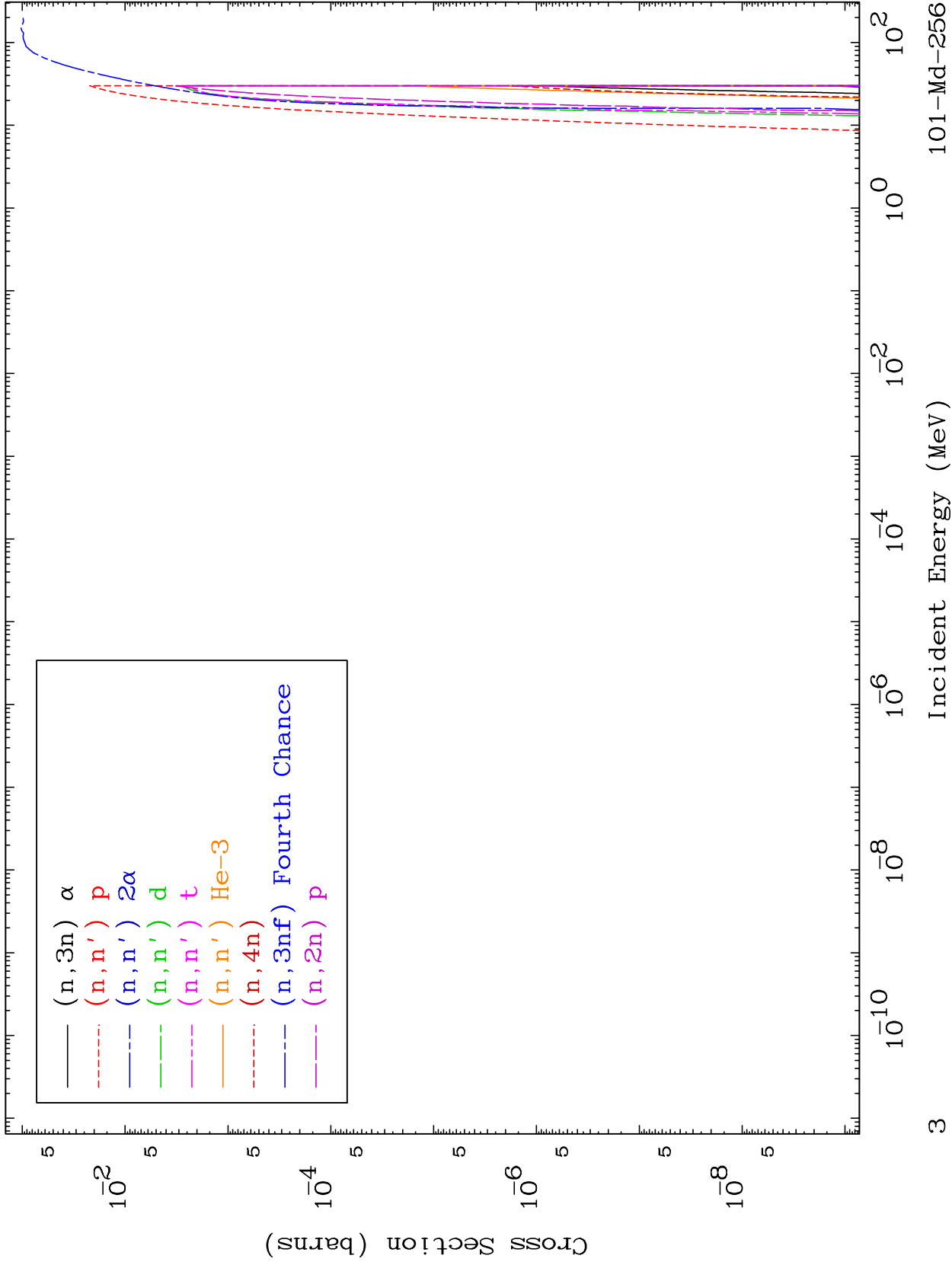


MAT 9957

Neutron Absorption
293 Kelvin Cross Sections

101-Md-256

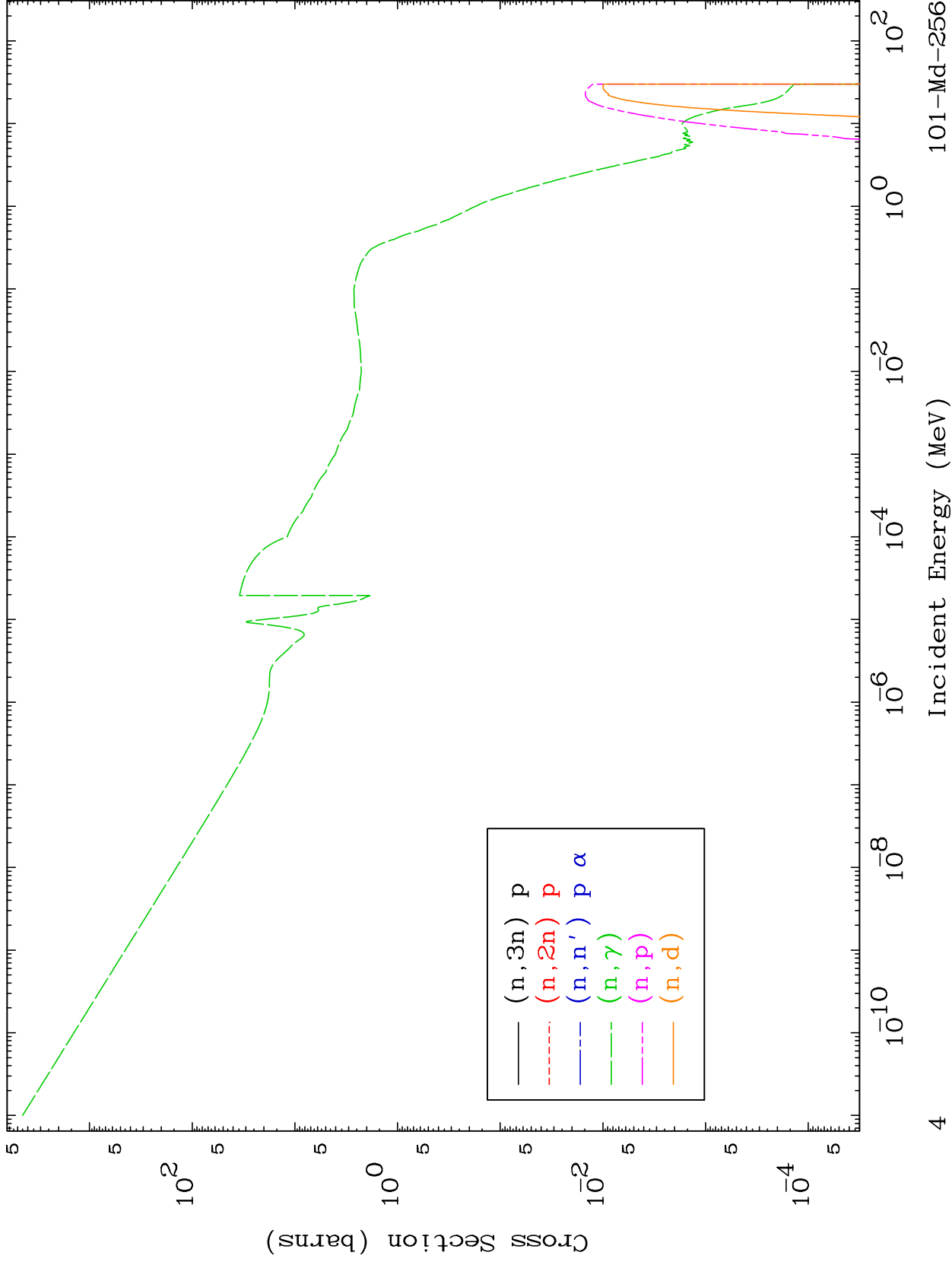


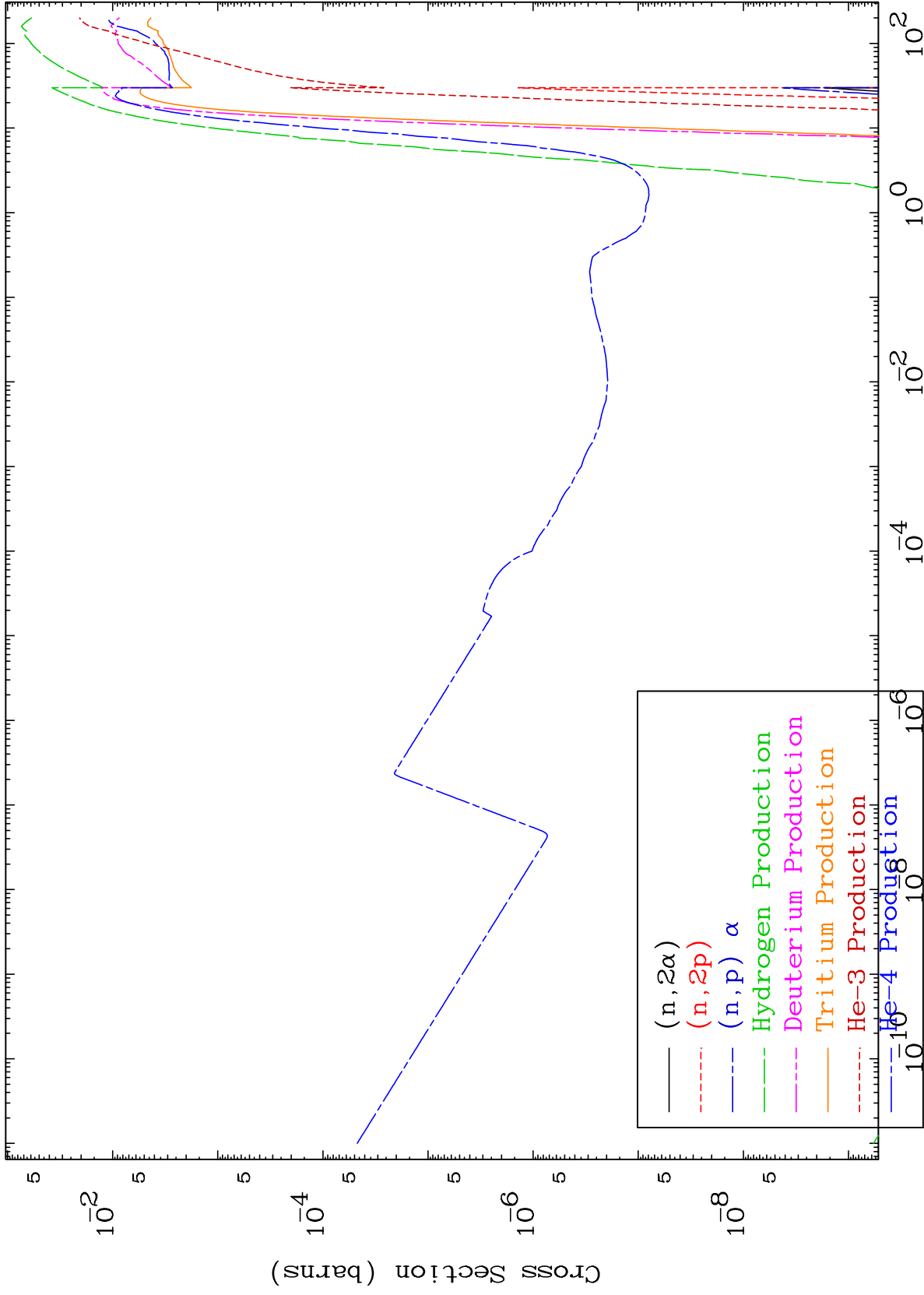


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Neutron Absorption
293 Kelvin Cross Sections

101-Md-256

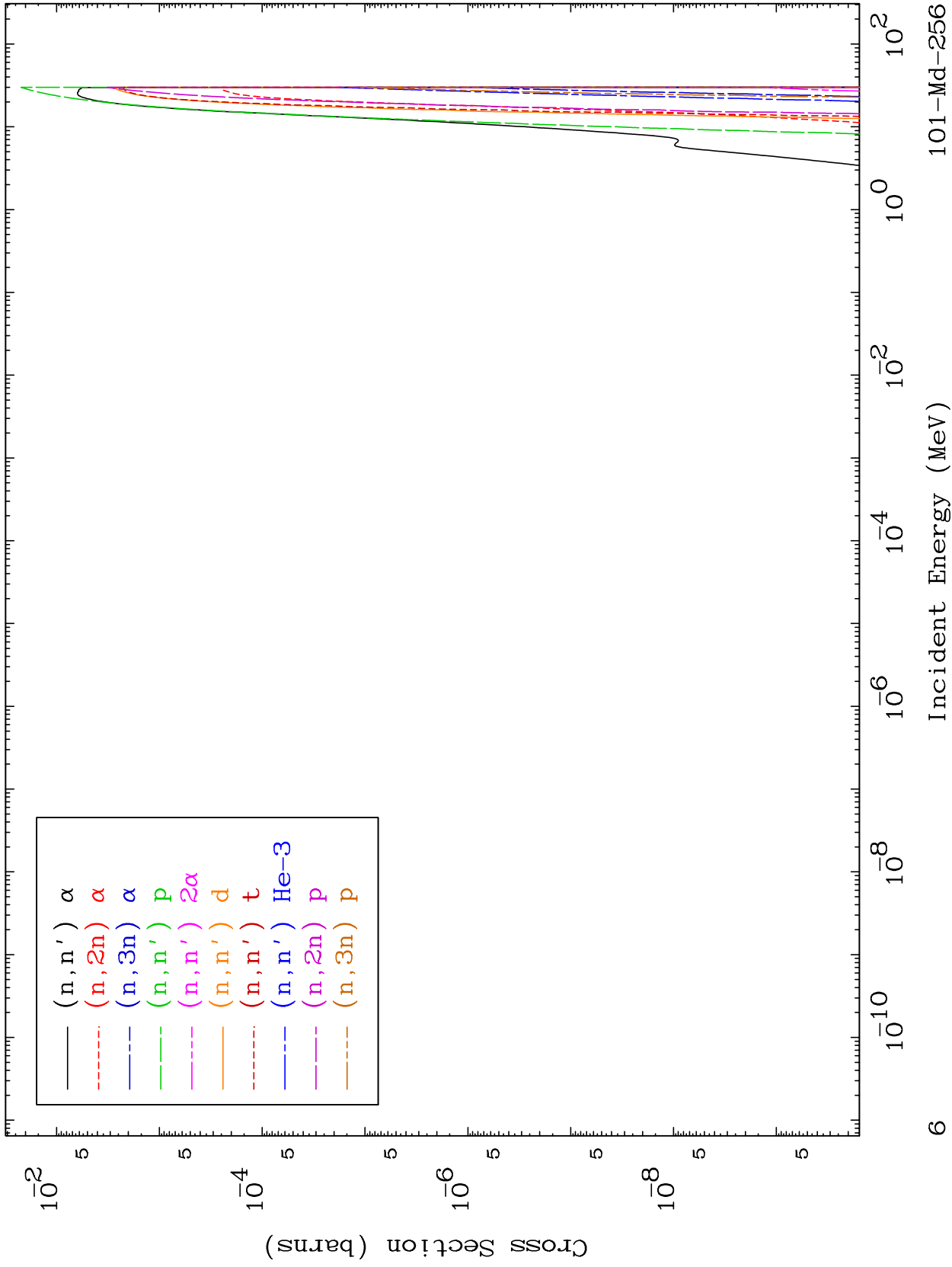




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Charged Particle
293 Kelvin Cross Sections

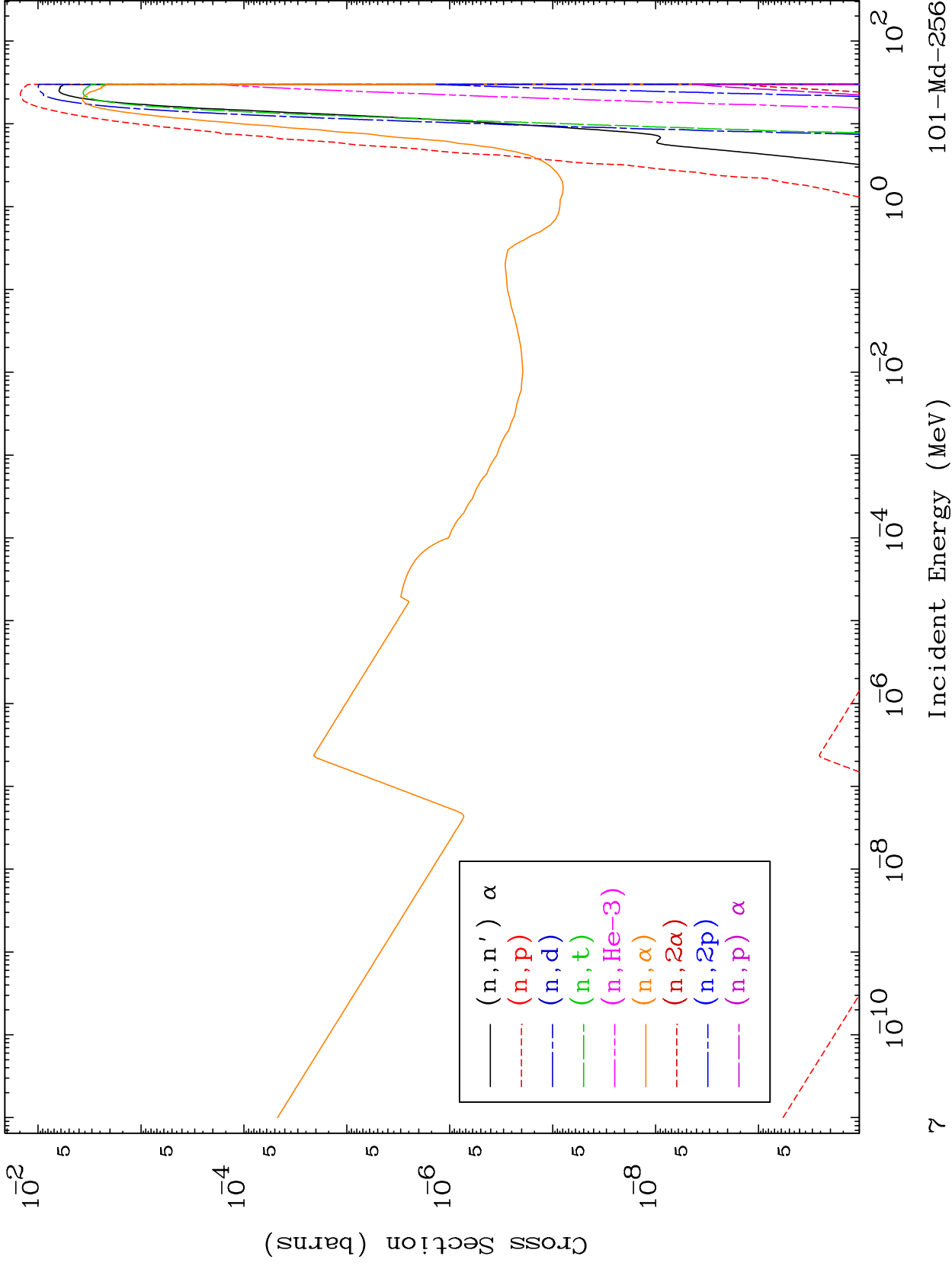
101-Md-256



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Charged Particle
293 Kelvin Cross Sections

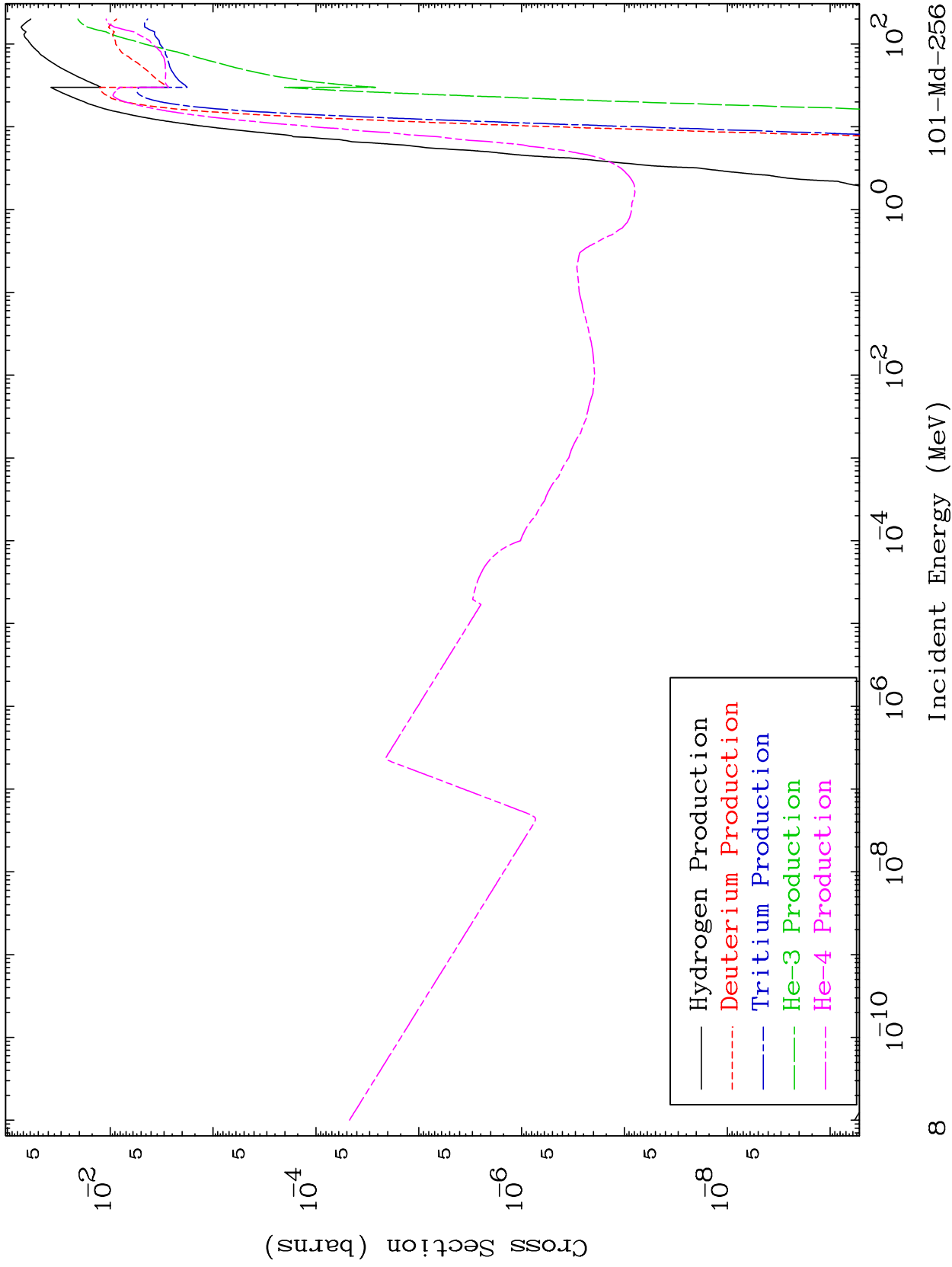
101-Md-256



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Particle Production
293 Kelvin Cross Sections

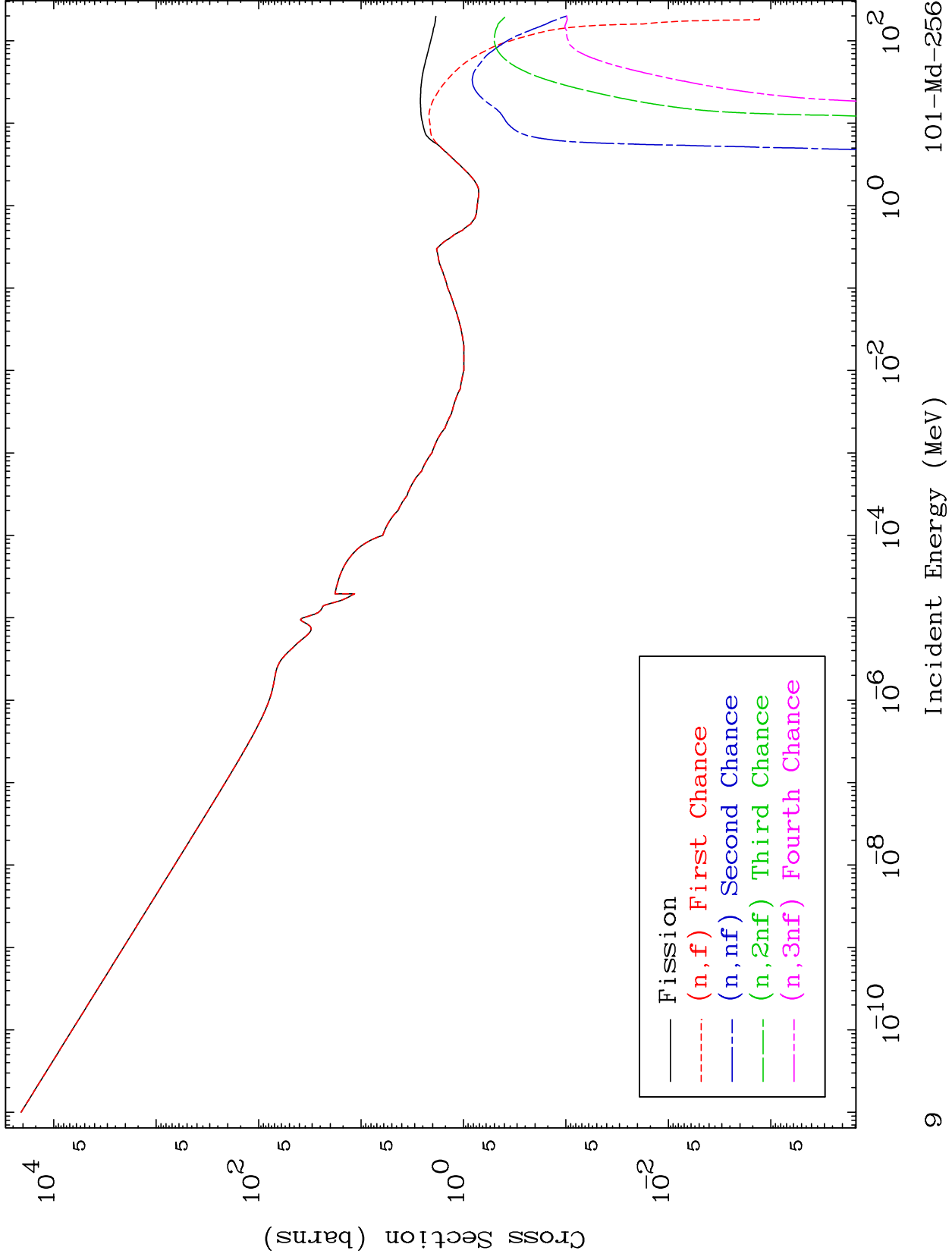
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Fission
293 Kelvin Cross Sections

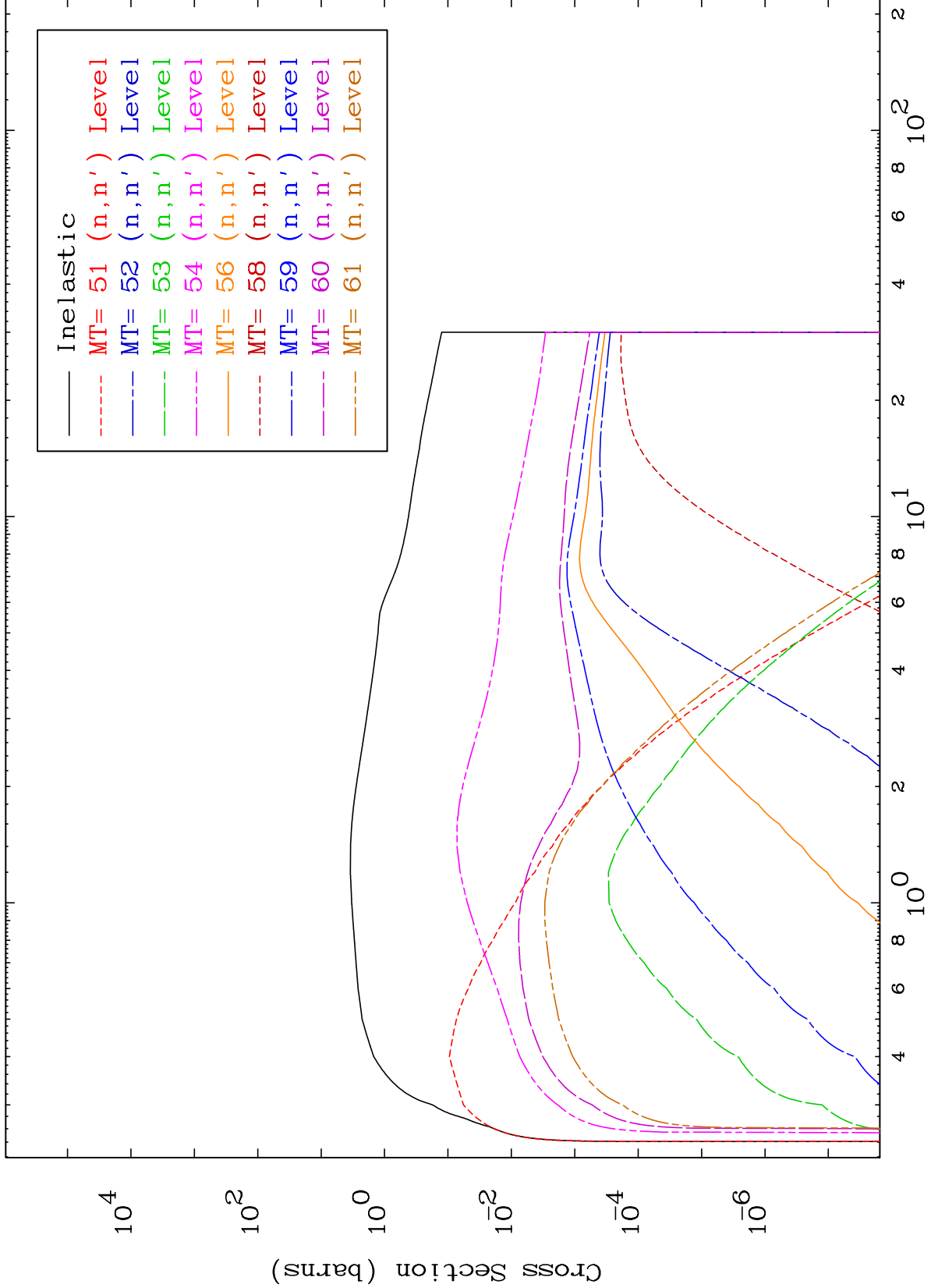
101-Md-256



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(n,n') Levels
293 Kelvin Cross Sections

101-Md-256



10

Incident Energy (MeV)

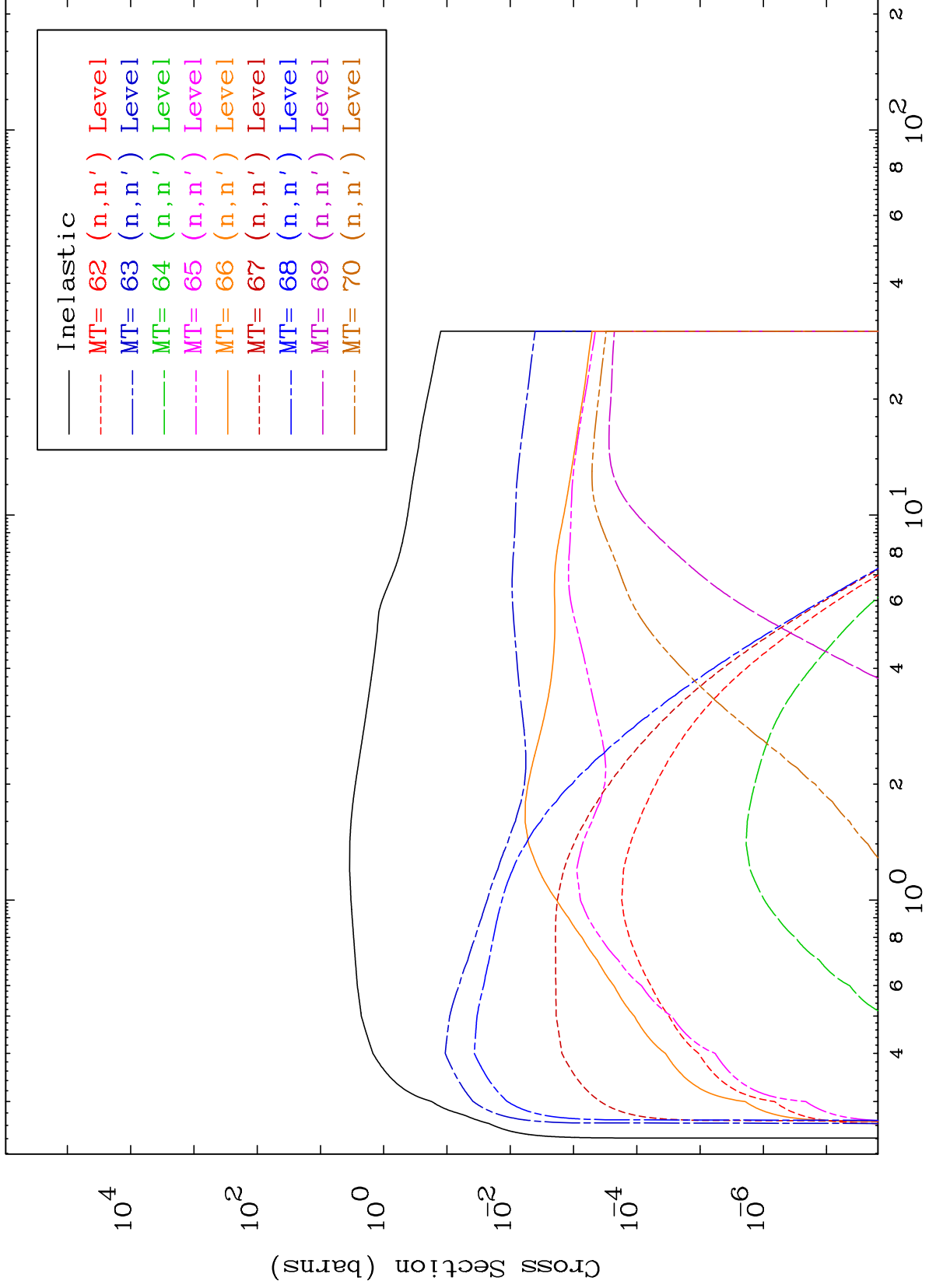
101-Md-256

MAT 9957

(n,n') Levels

101-Md-256

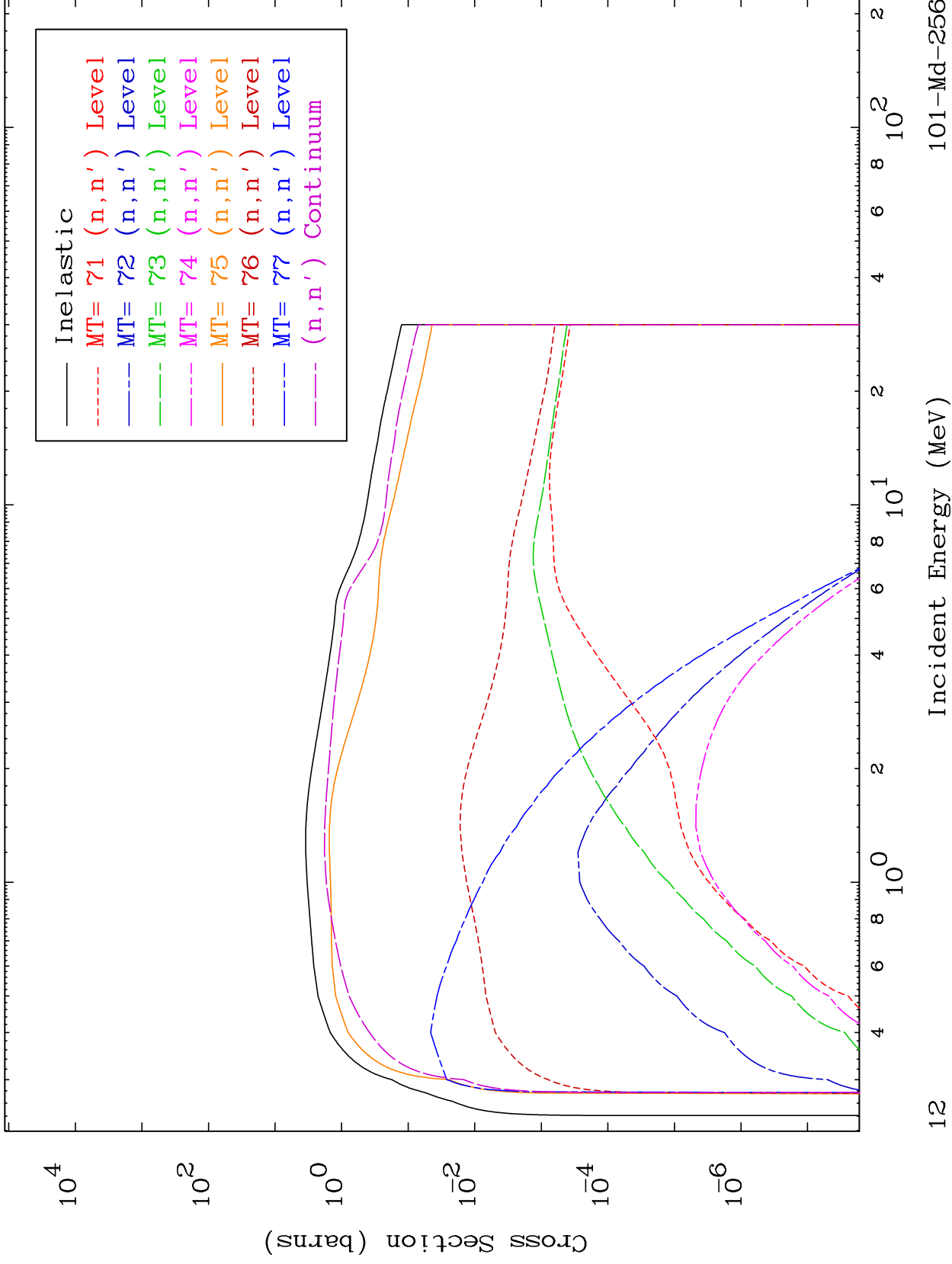
293 Kelvin Cross Sections



MAT 9957

(n,n') Levels
293 Kelvin Cross Sections

101-Md-256



12

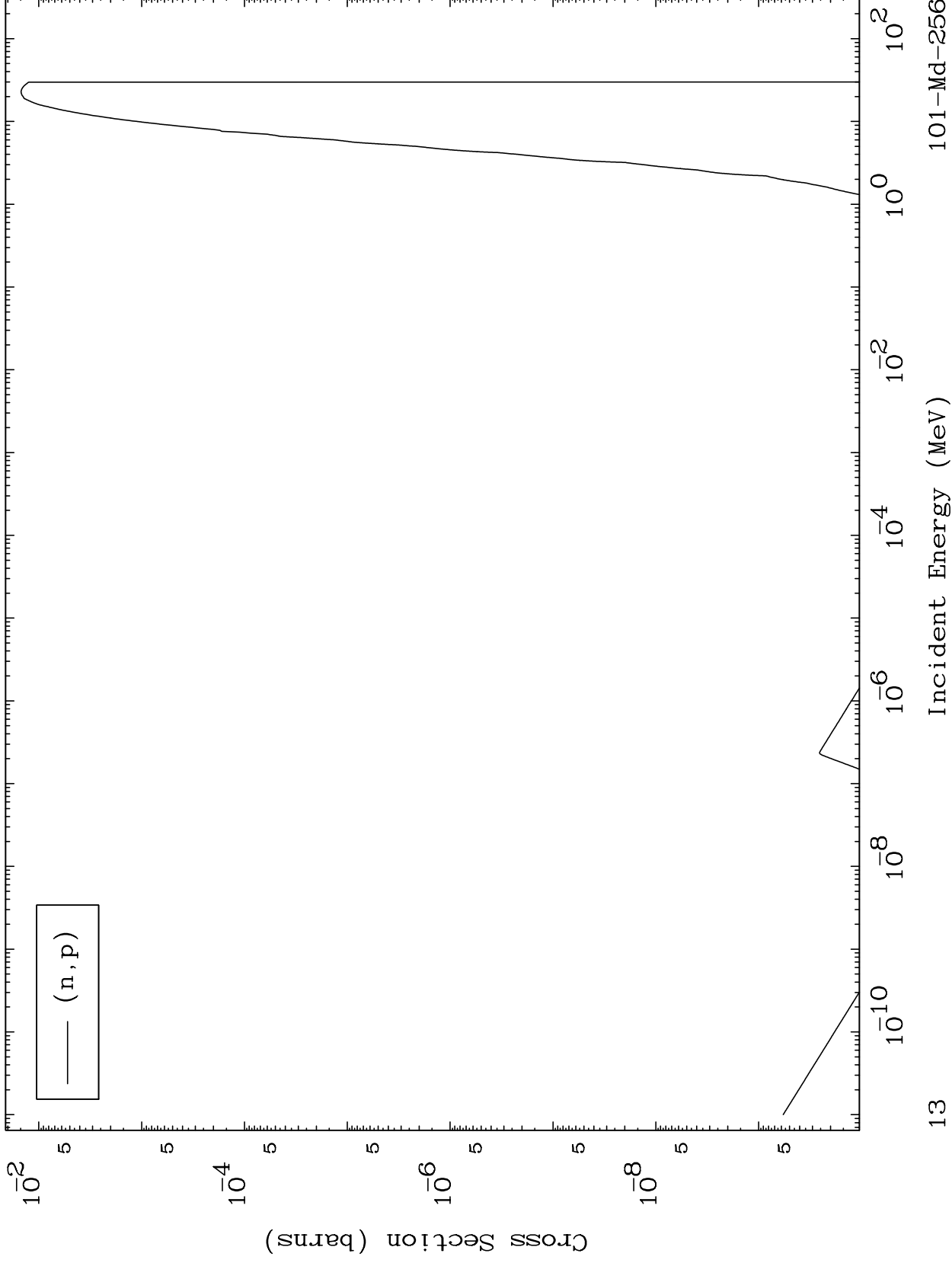
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,p) Levels
293 Kelvin Cross Sections

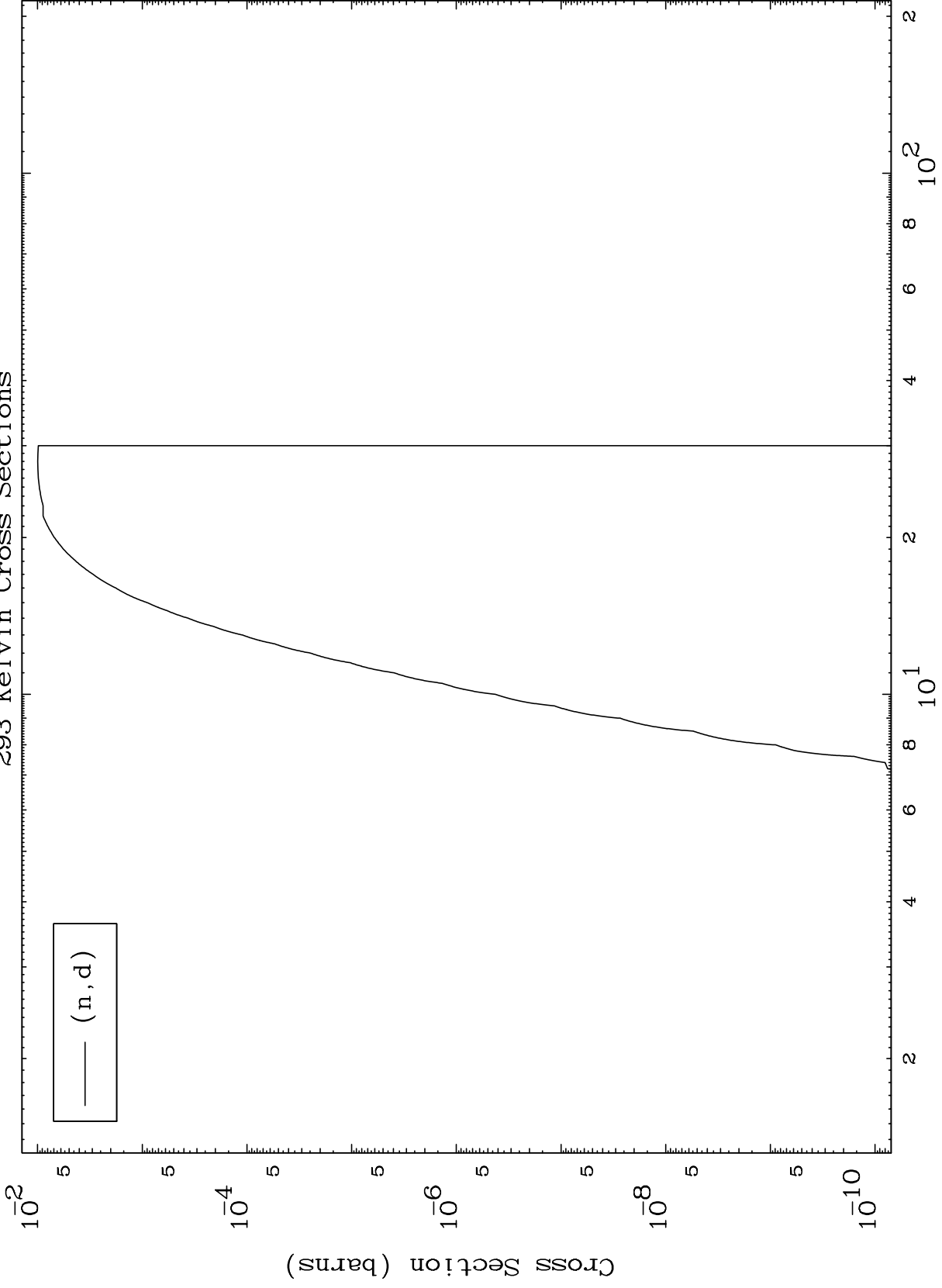
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MAT 9957

(n,d) Levels
293 Kelvin Cross Sections

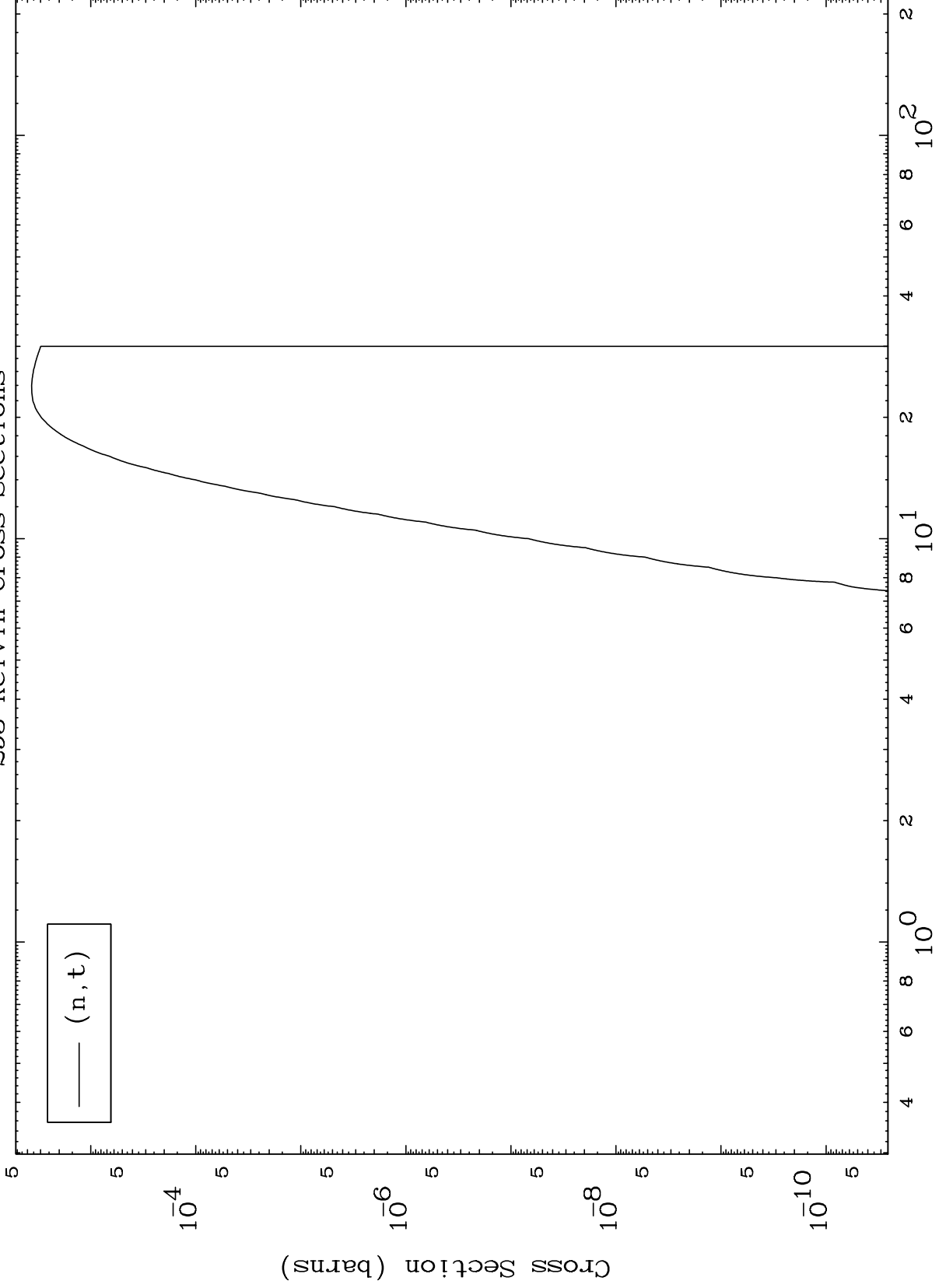
101-Md-256



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(n,t) Levels
293 Kelvin Cross Sections

101-Md-256



15

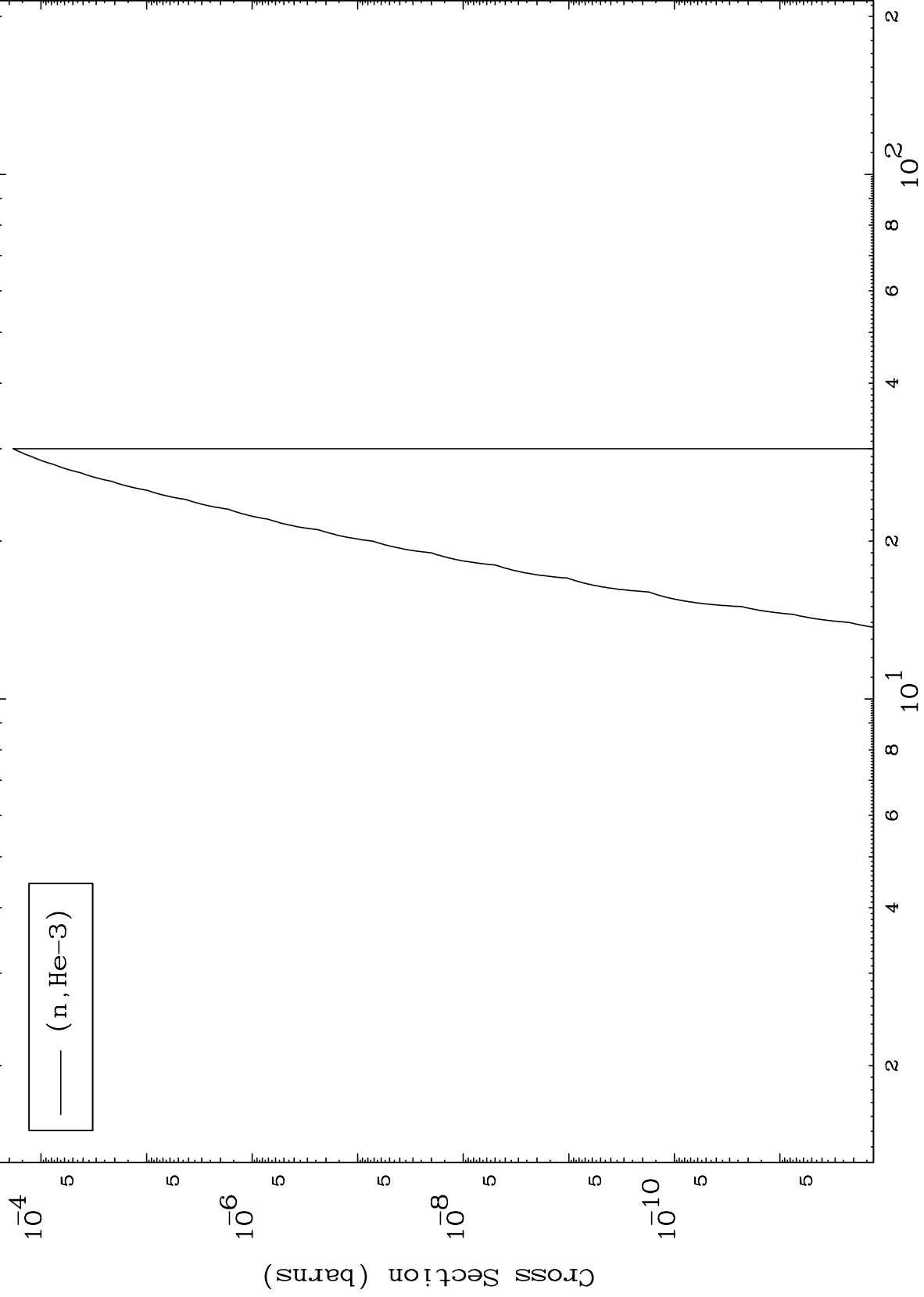
Incident Energy (MeV)

101-Md-256

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(n,He3) Levels
293 Kelvin Cross Sections

101-Md-256



16

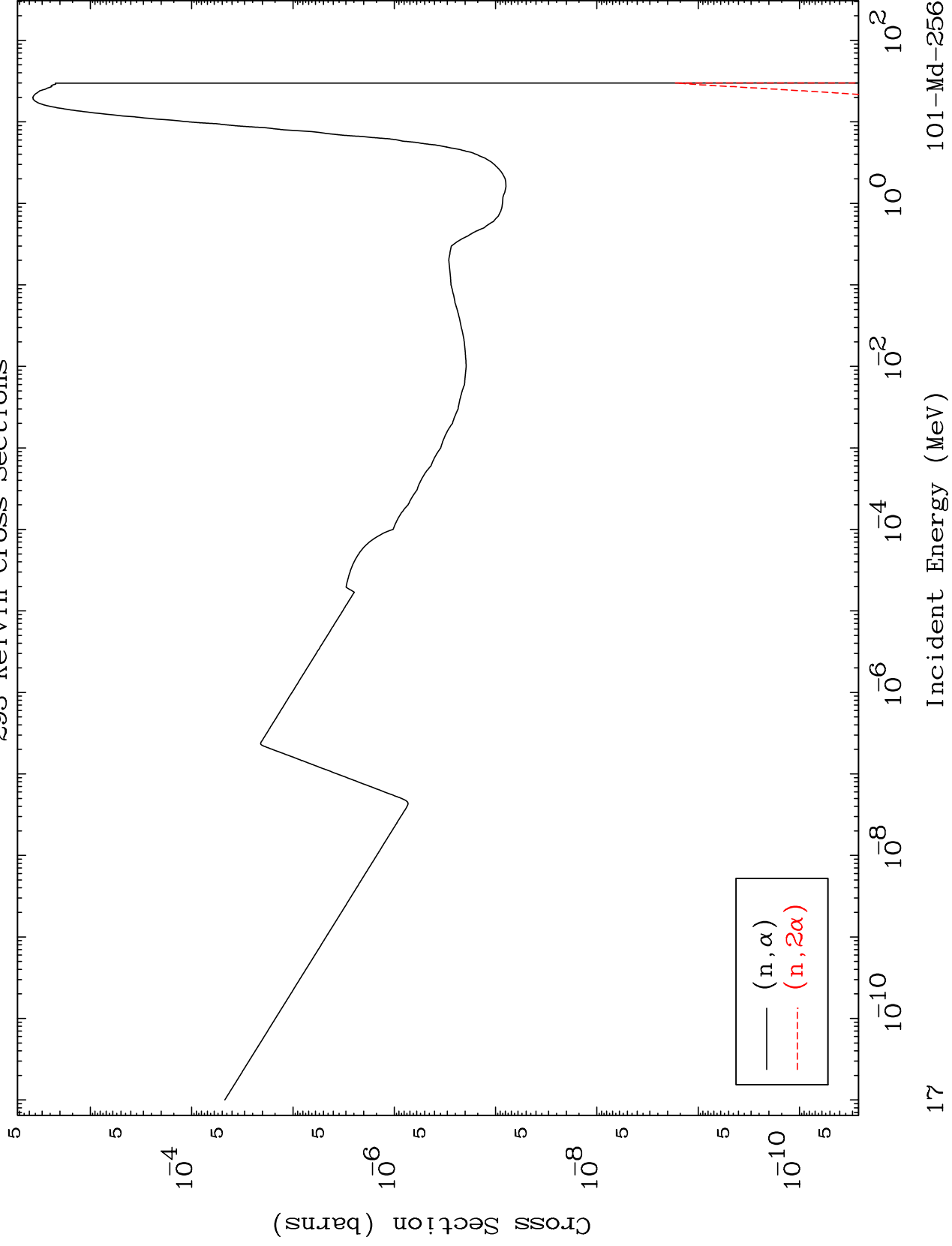
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,α) Levels
293 Kelvin Cross Sections

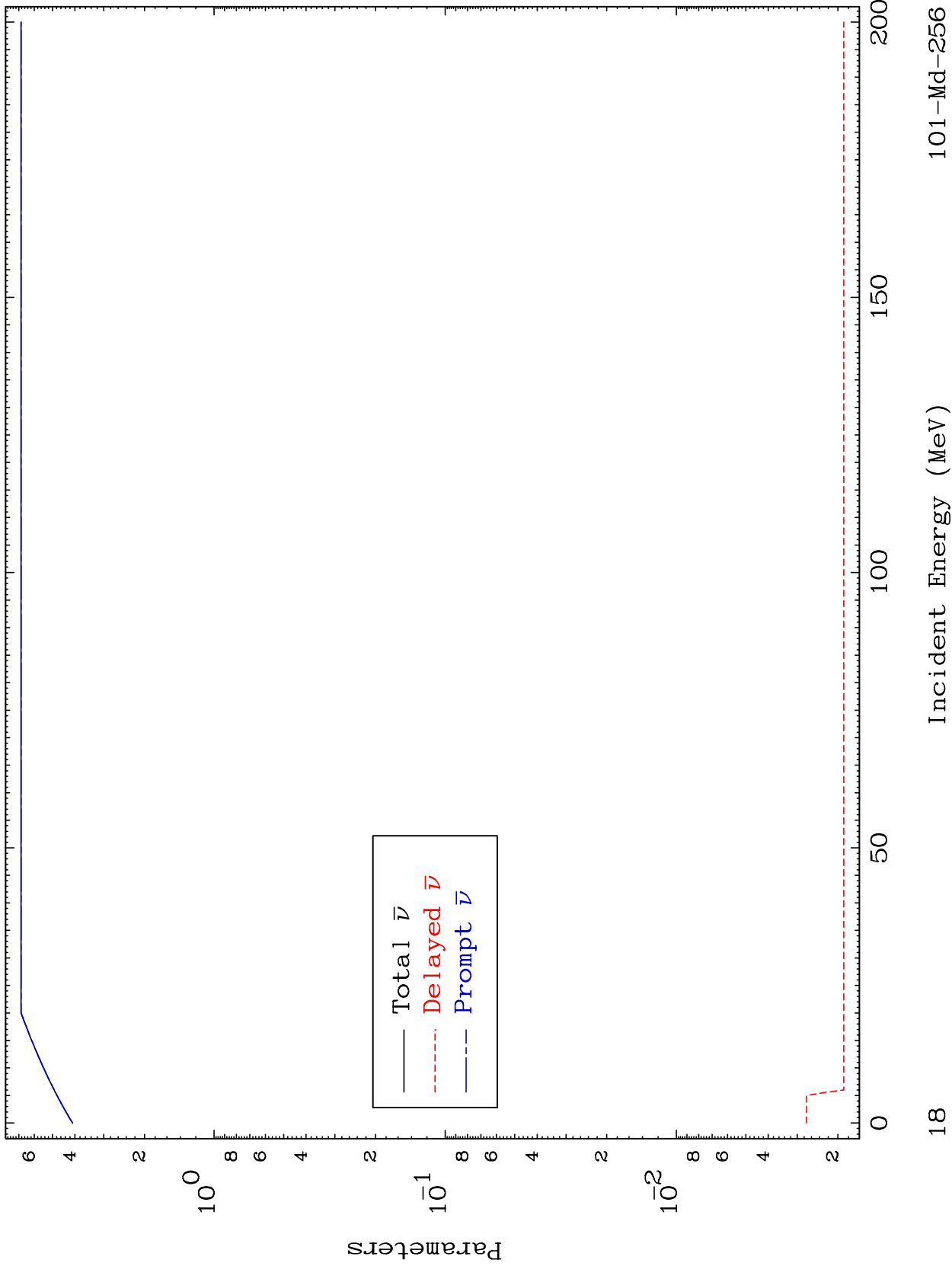
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Energy Release
Parameters

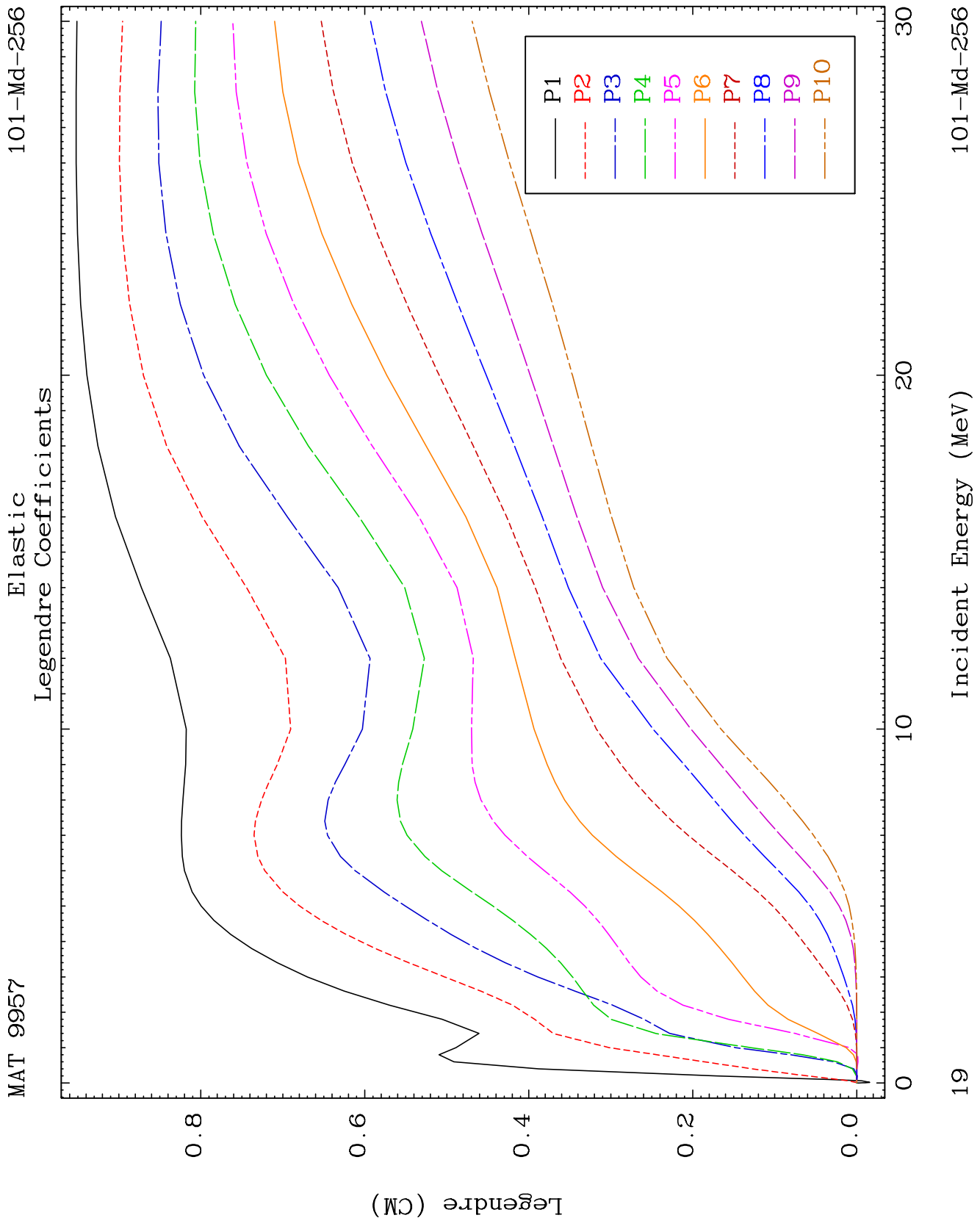
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Incident Energy (MeV)

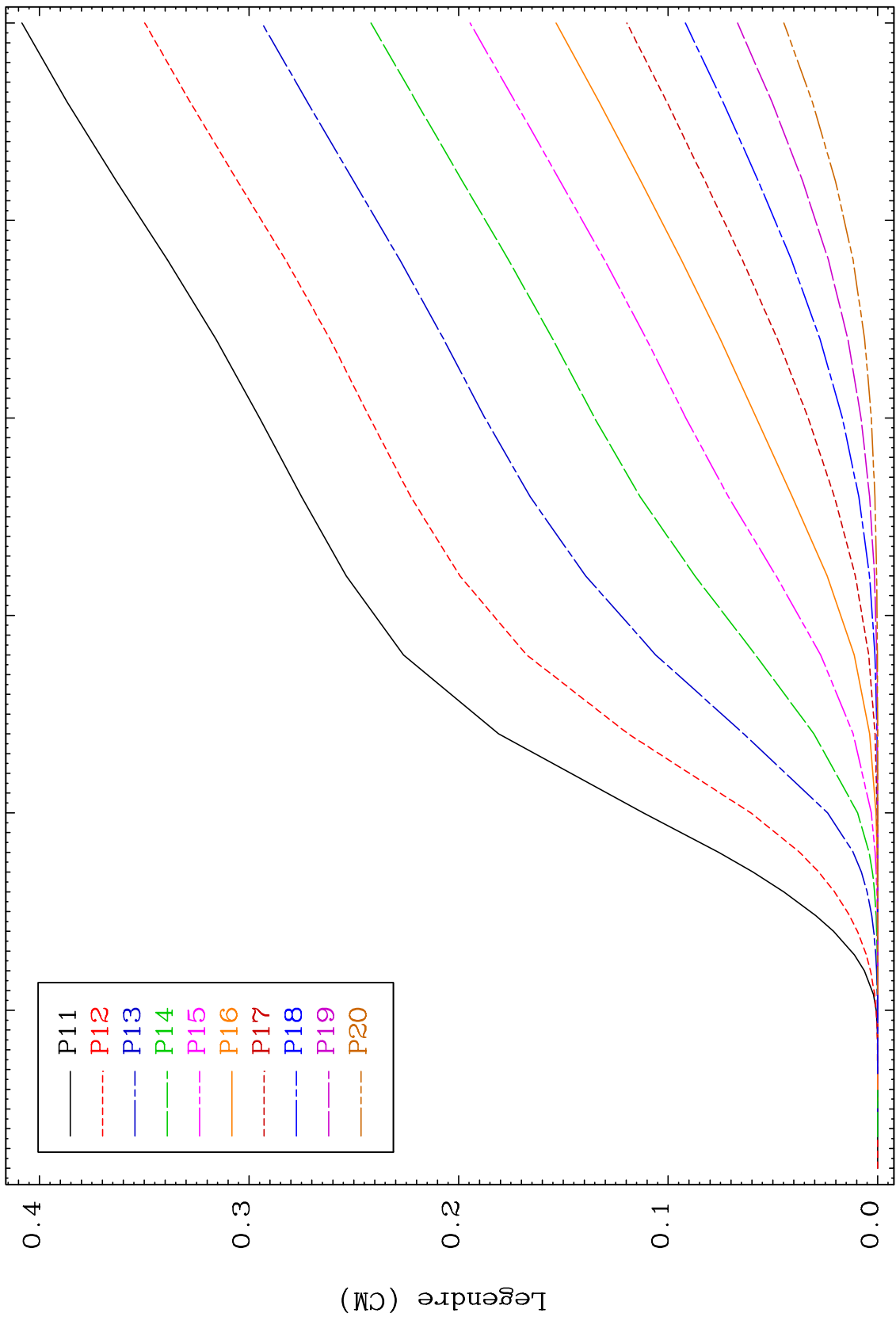
101-Md-256



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Elastic Legendre Coefficients

101-Md-256



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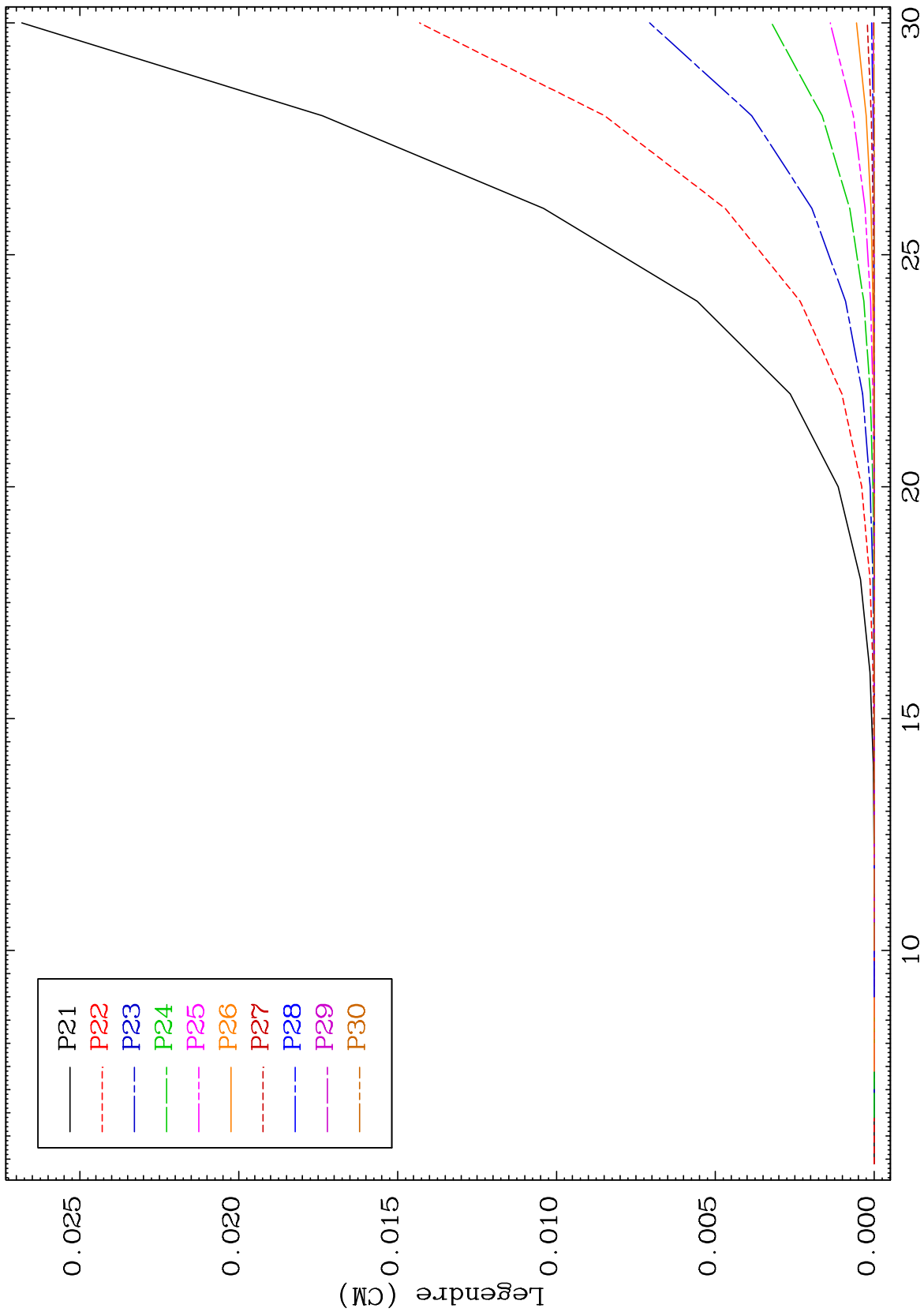
Incident Energy (MeV)

101-Md-256

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Elastic Legendre Coefficients

101-Md-256



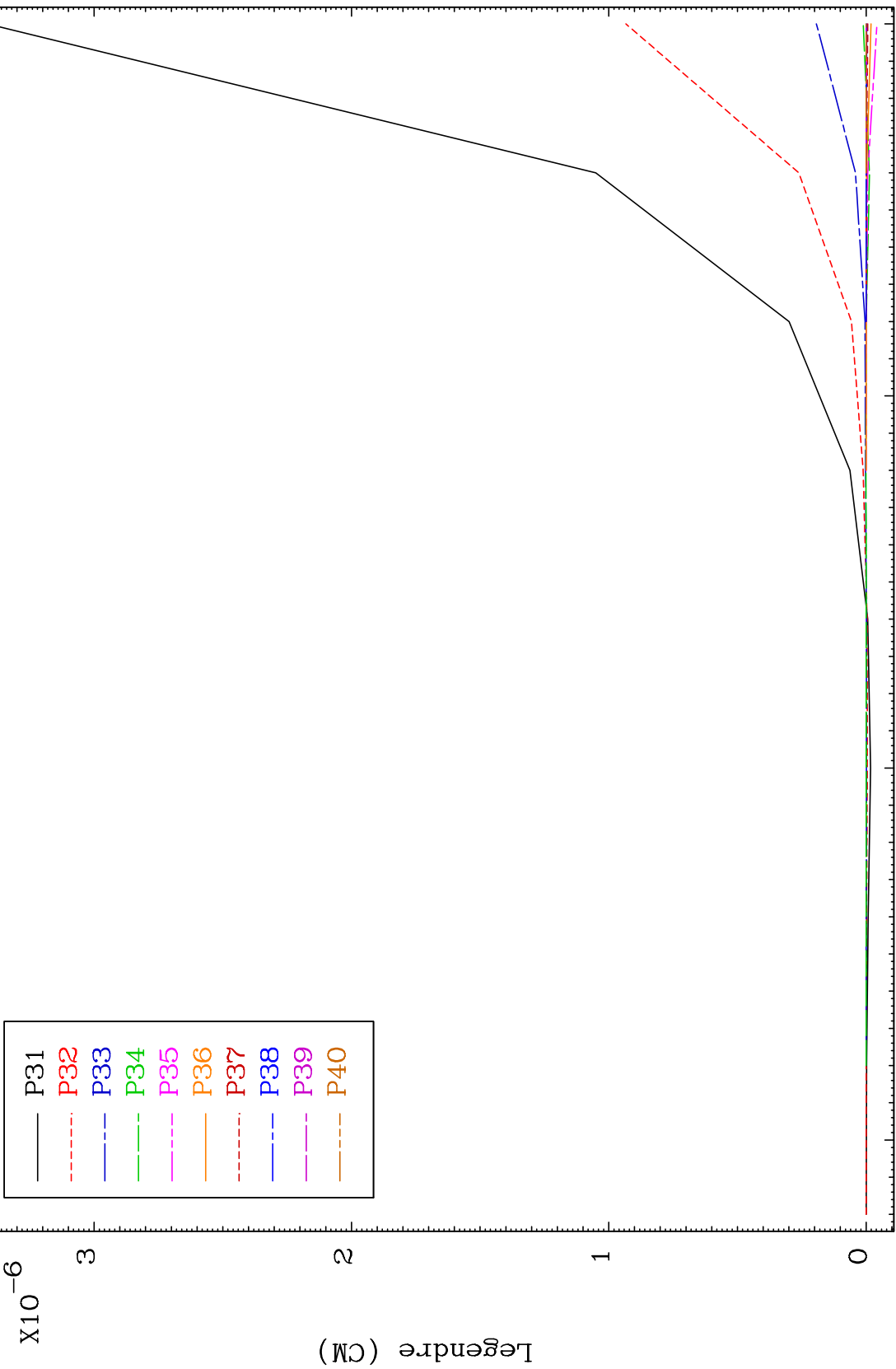
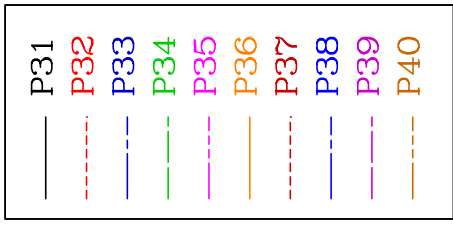
21

101-Md-256

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Elastic Legendre Coefficients

101-Md-256



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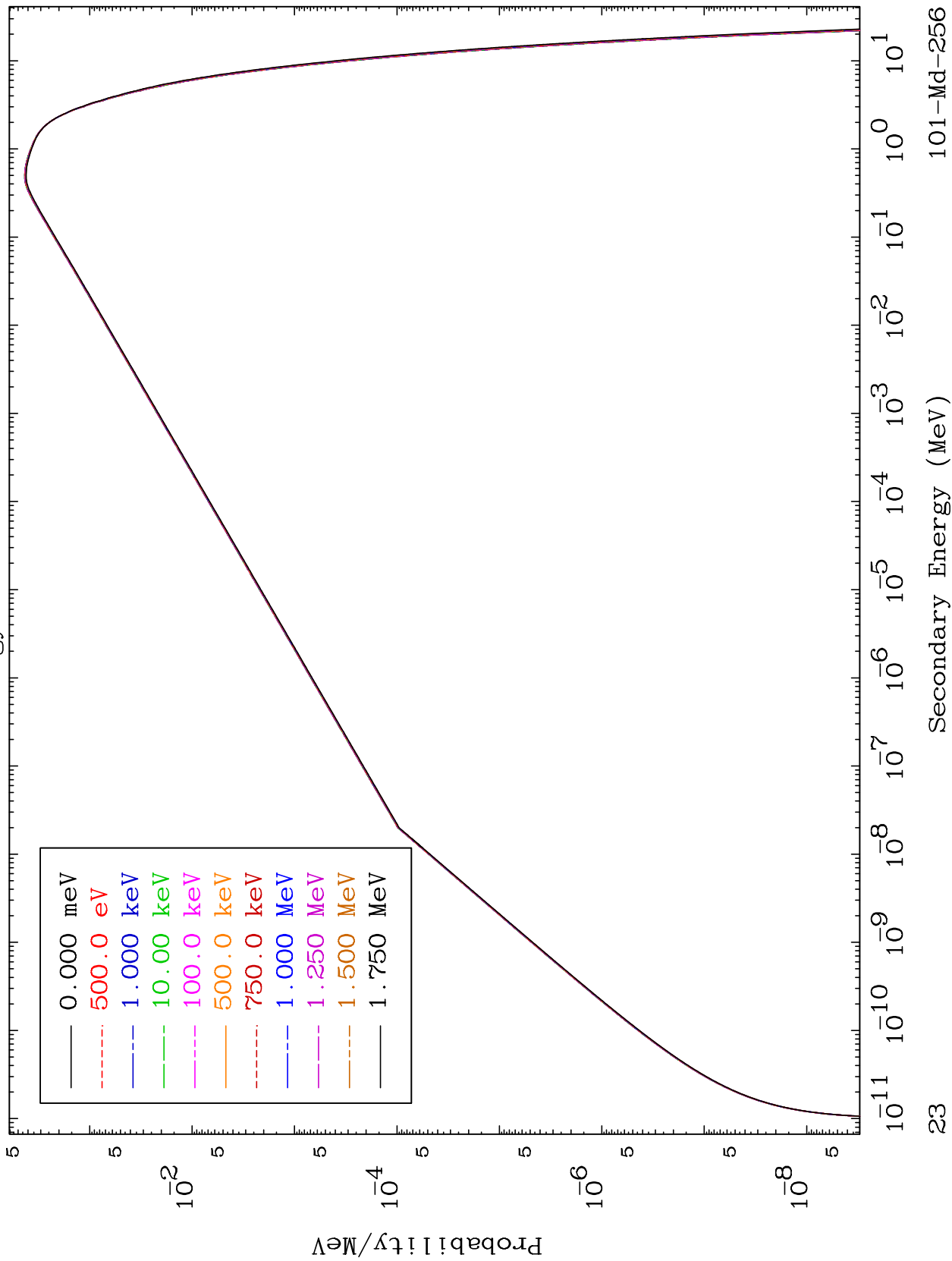
Incident Energy (MeV)

101-Md-256

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Fission Energy Distributions

101-Md-256



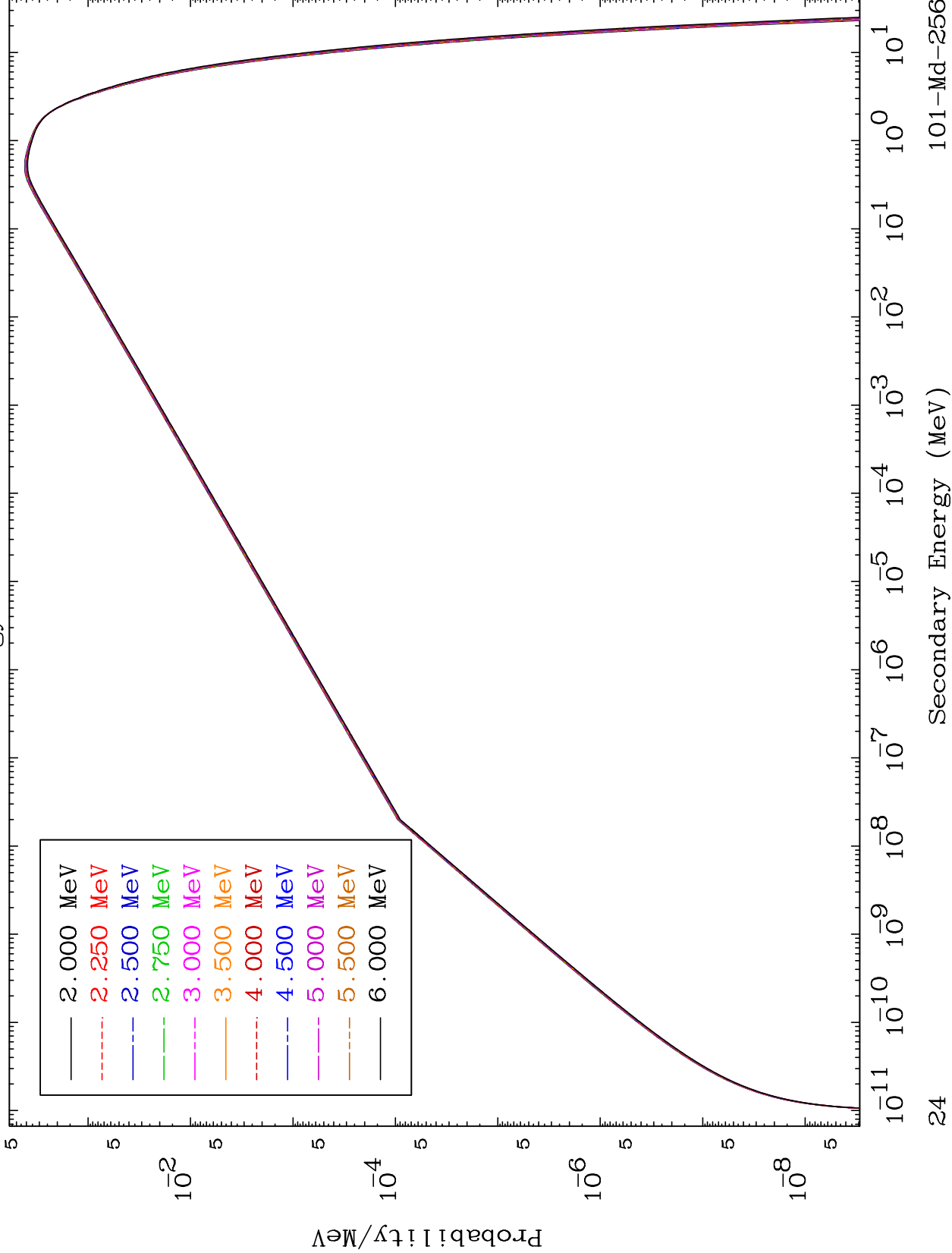
101-Md-256

Secondary Energy (MeV)

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Fission Energy Distributions

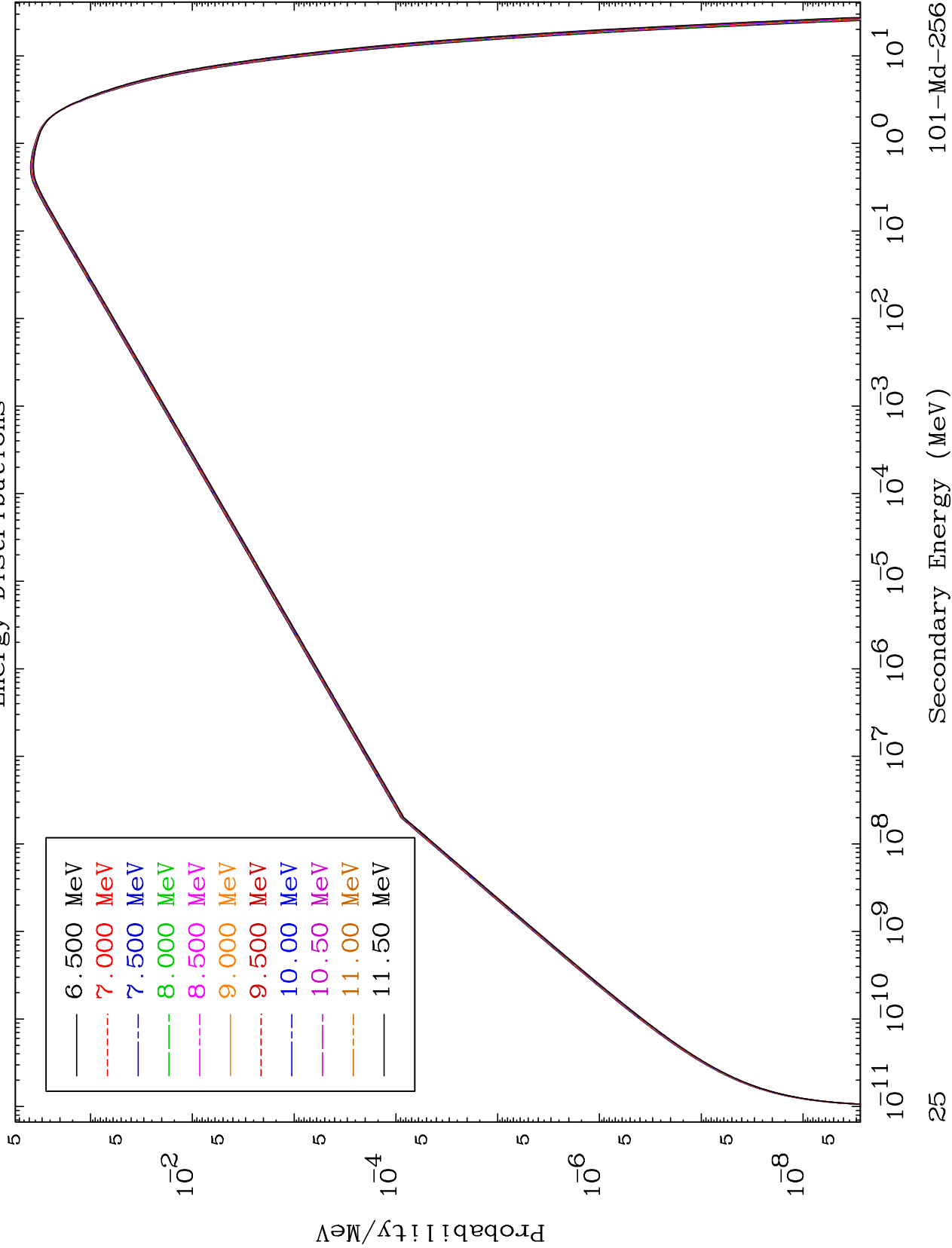
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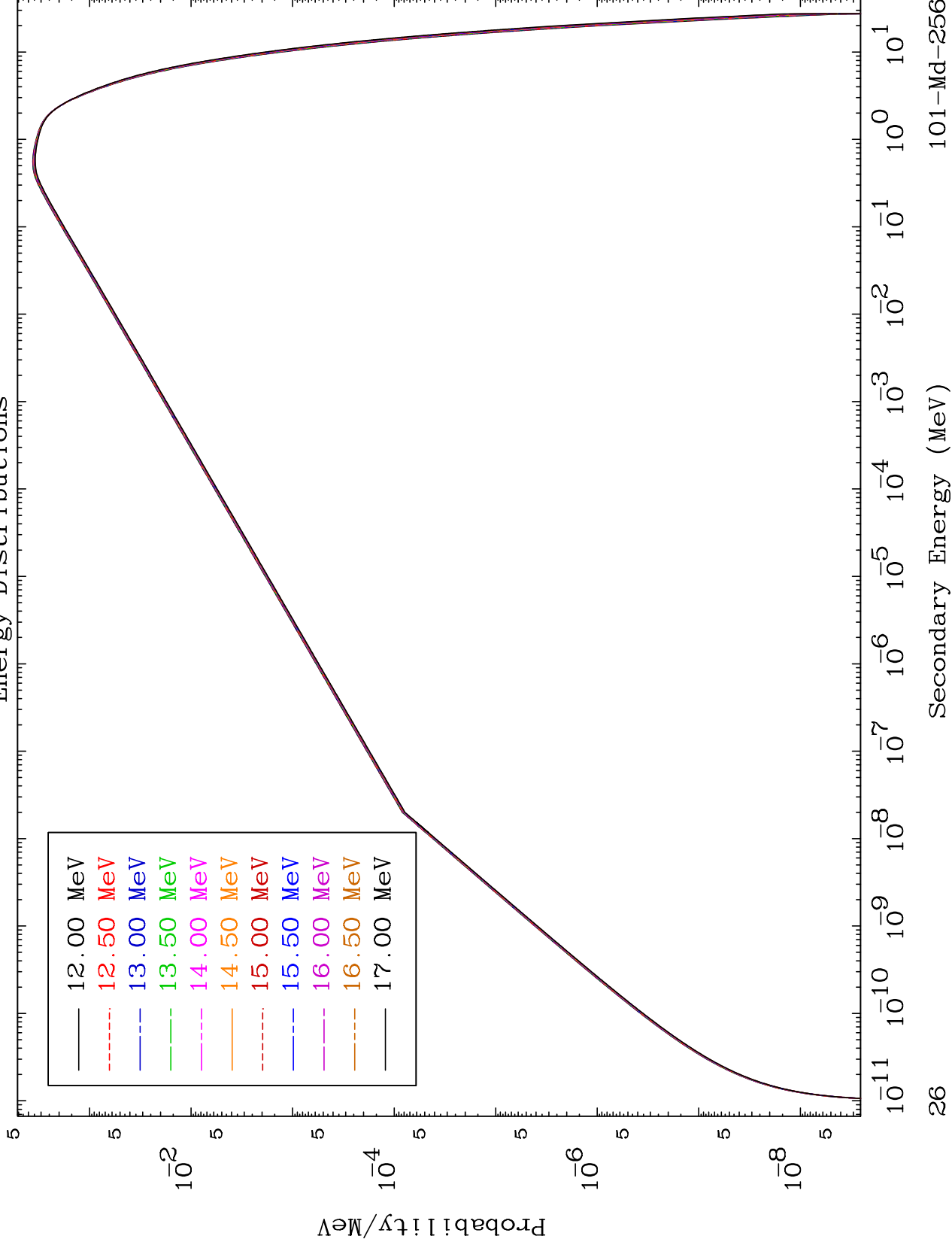


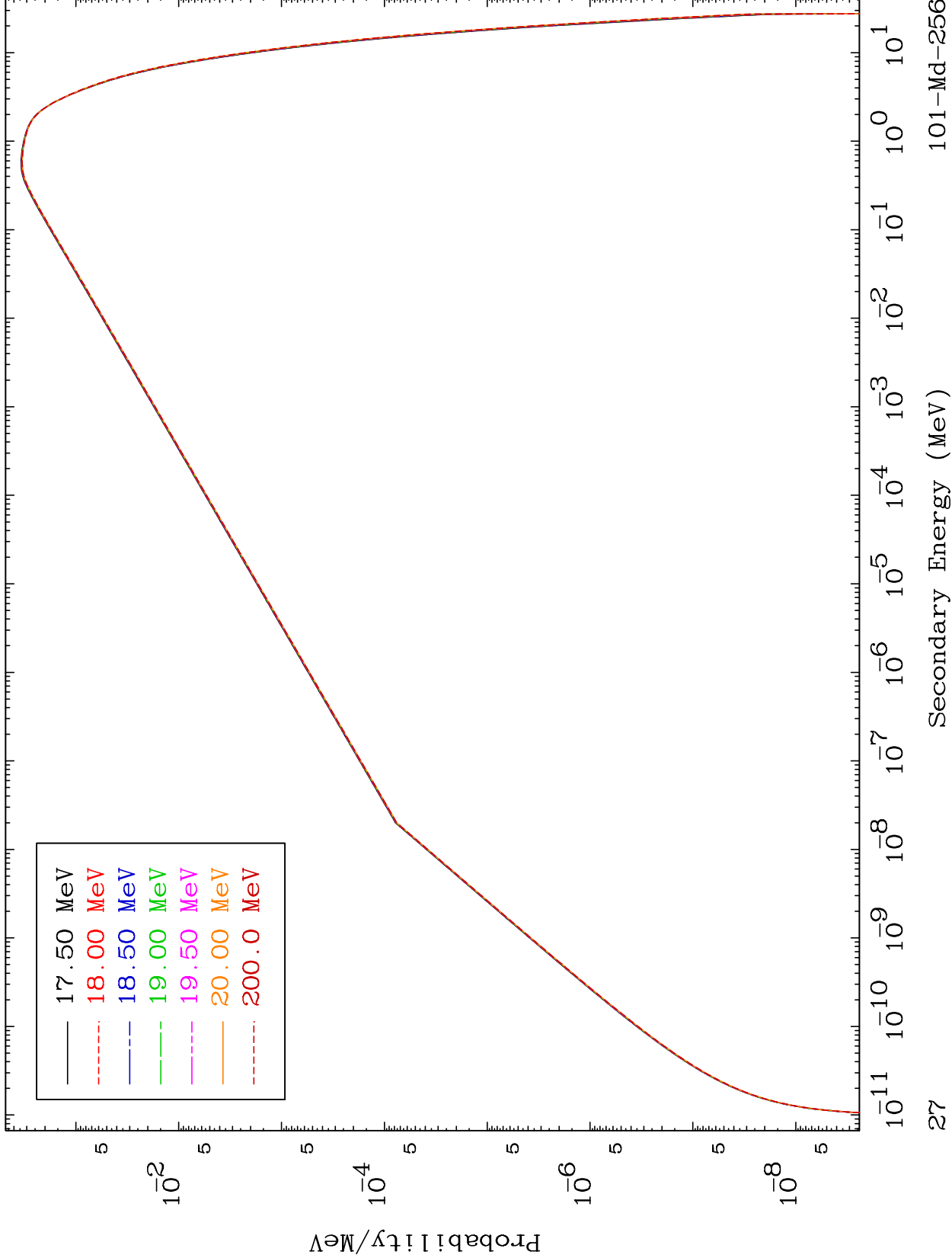
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Secondary Energy (MeV)

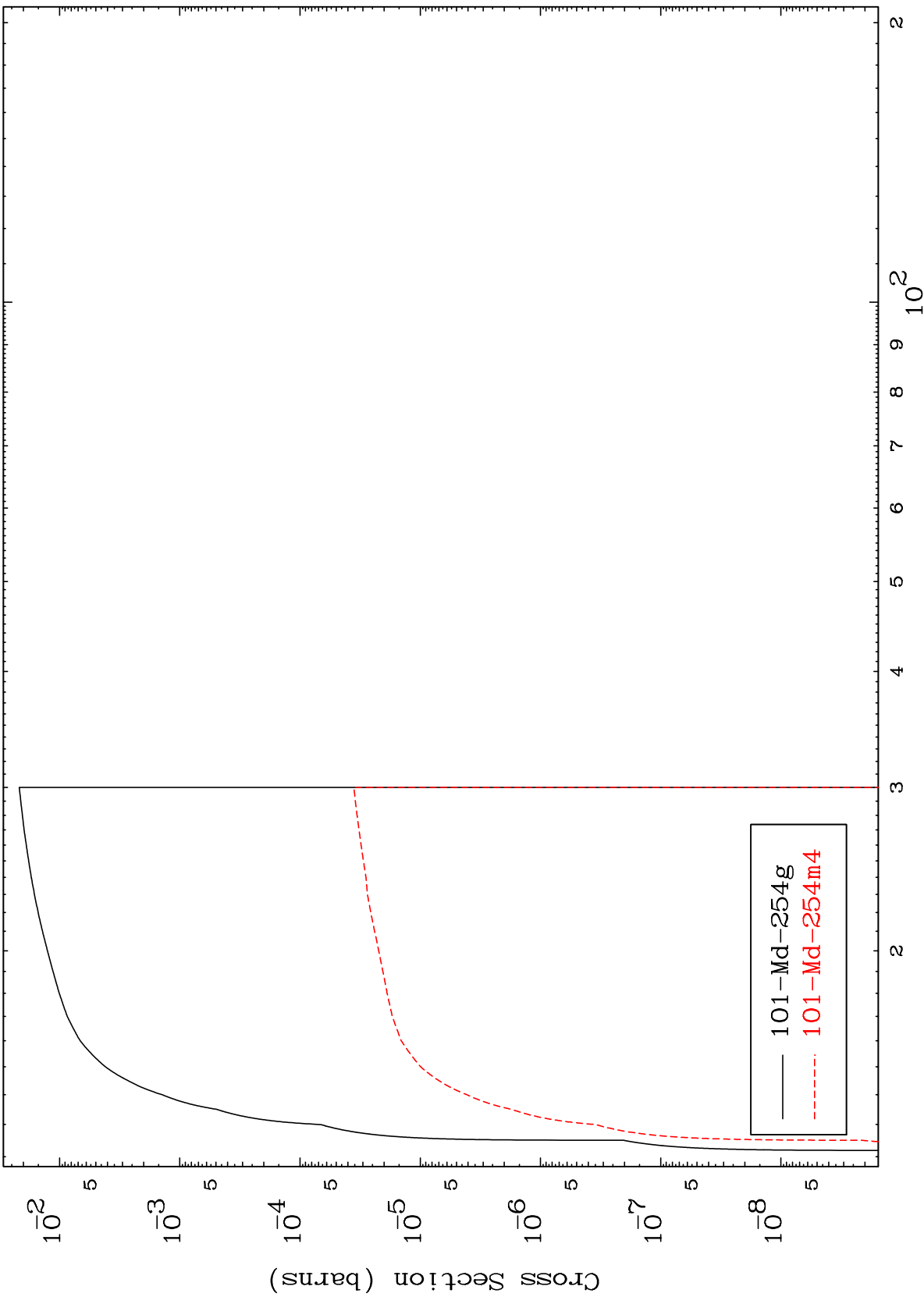
Fission Energy Distributions







Radionuclide Production Cross Section
(n,3n)

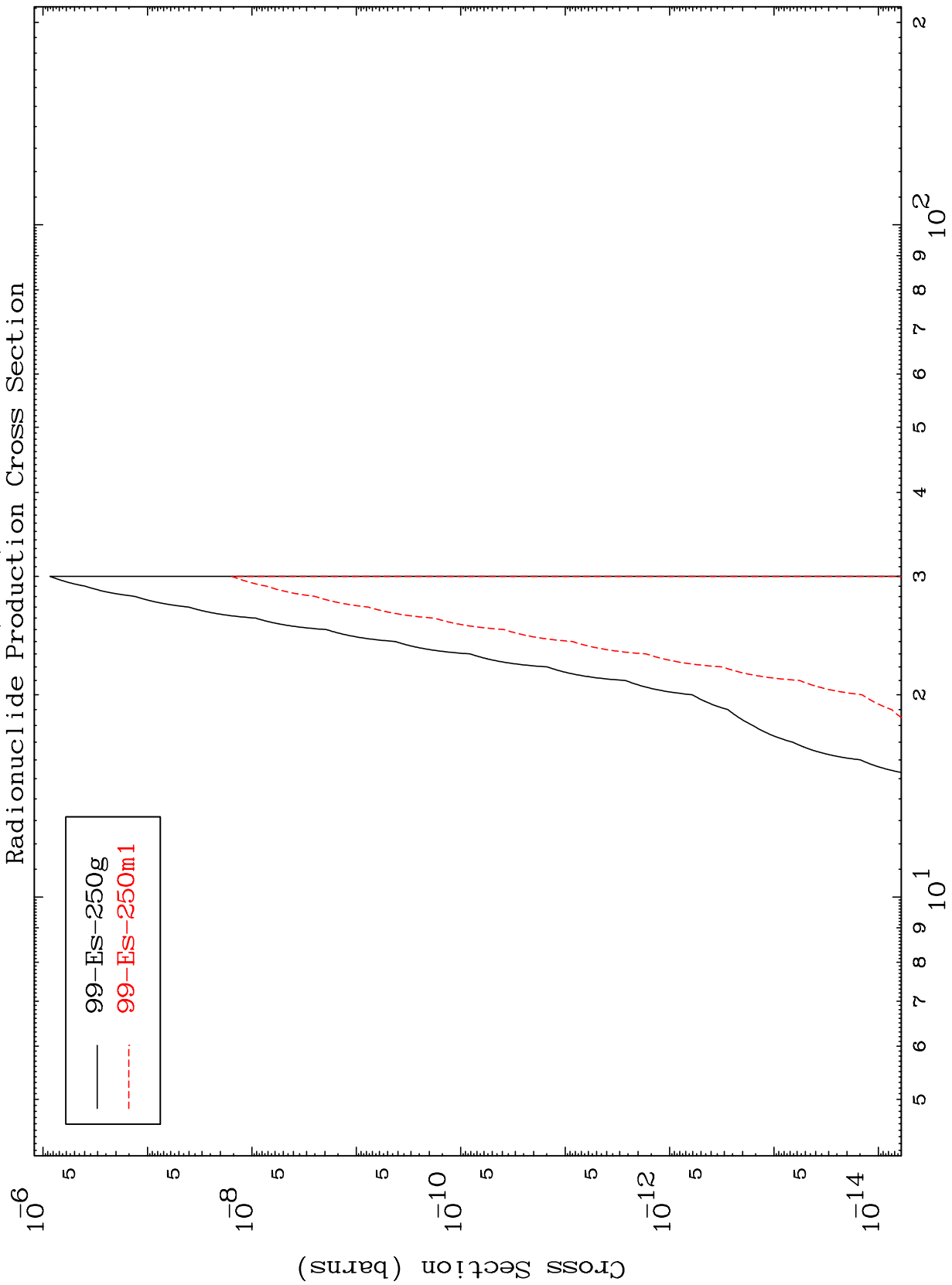


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(n,3n) α

101-Md-256

Radionuclide Production Cross Section



99-Es-250g
99-Es-250m1

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Incident Energy (MeV)

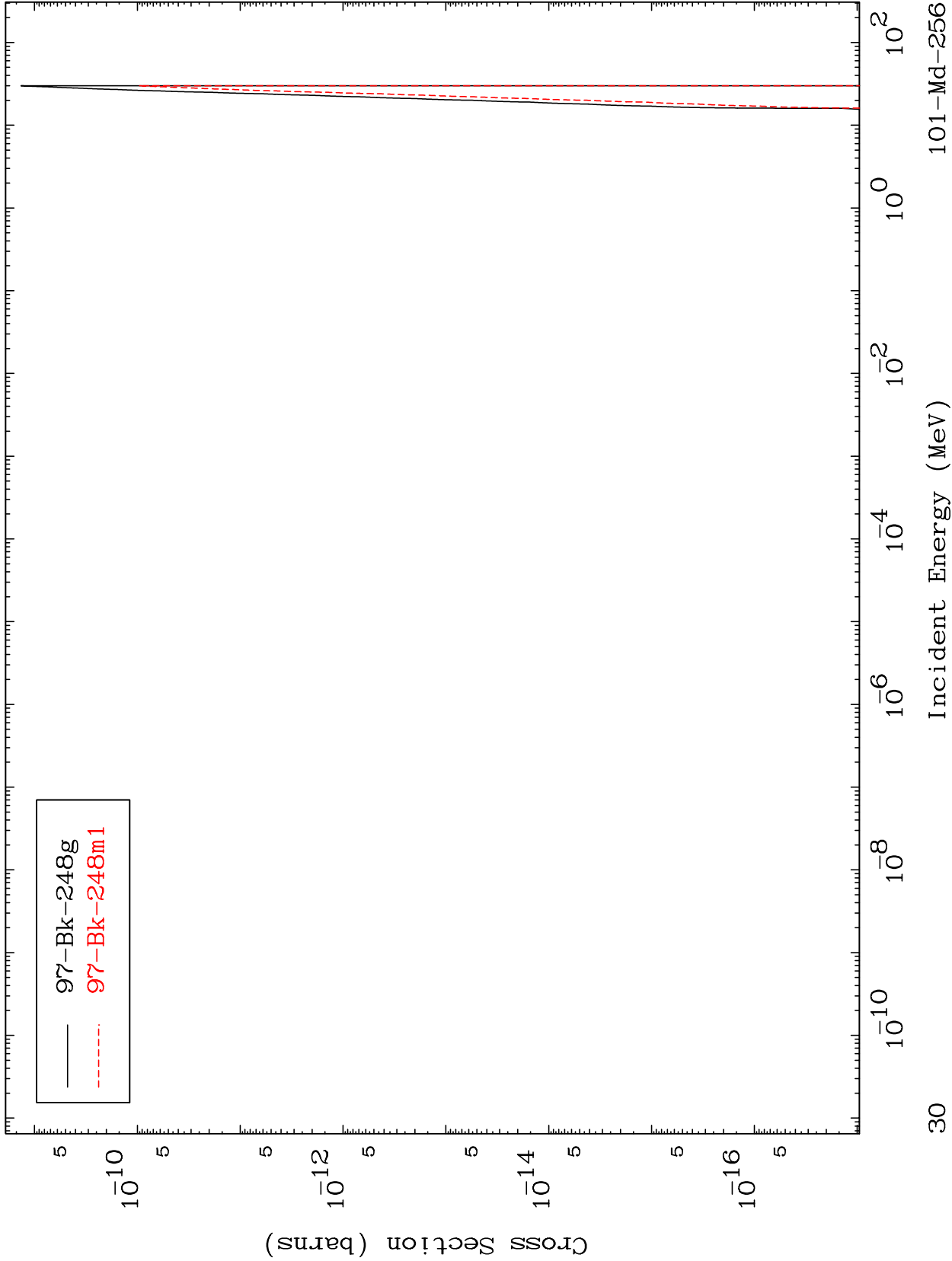
101-Md-256

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(n,n') 2 α

101-Md-256

Radionuclide Production Cross Section

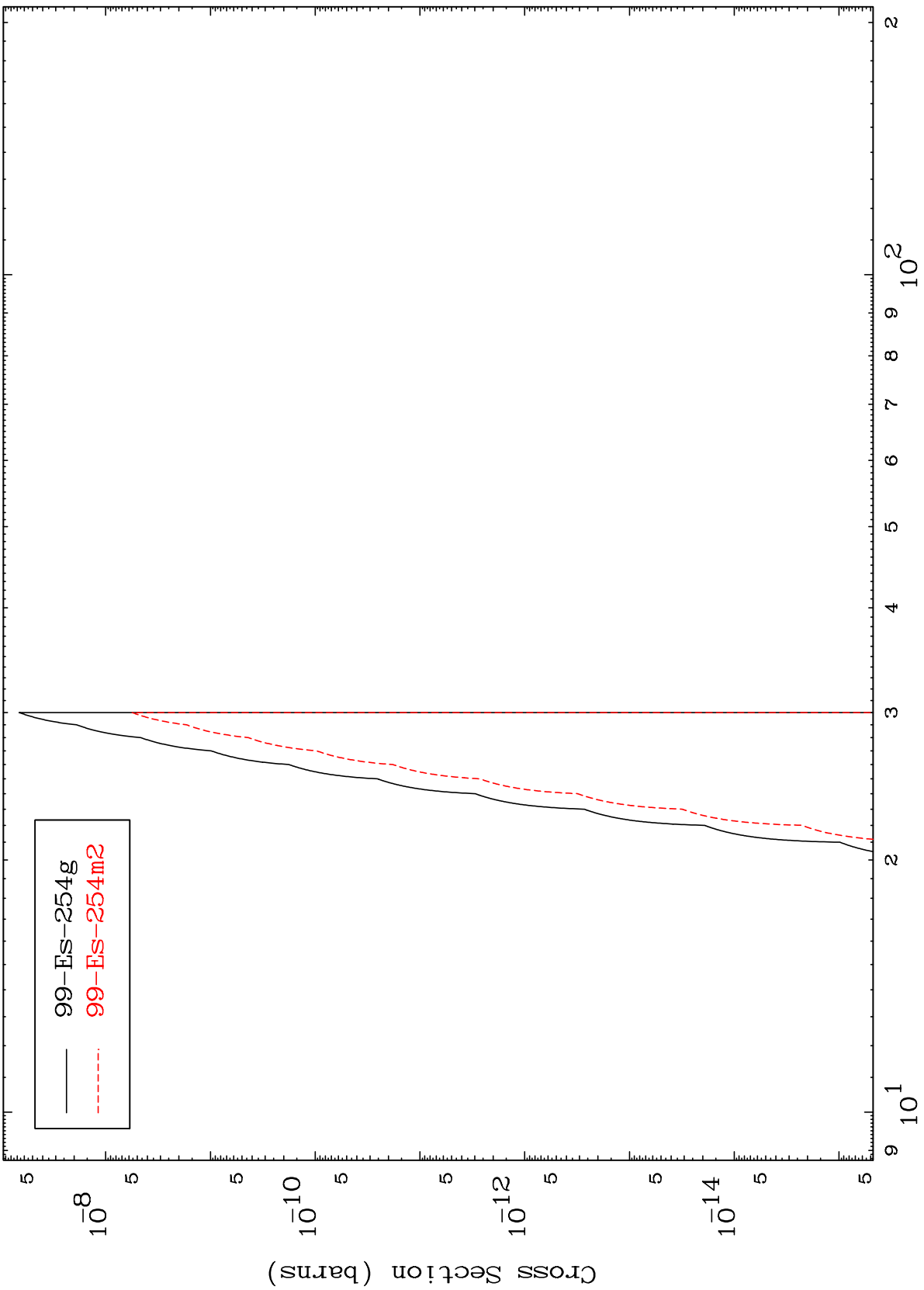


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(n,2n) p

101-Md-256

Radionuclide Production Cross Section



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Incident Energy (MeV)

101-Md-256

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(n,He-3)

101-Md-256

Radionuclide Production Cross Section

