

Program EVALPLOT
(Version 2021-1)

by

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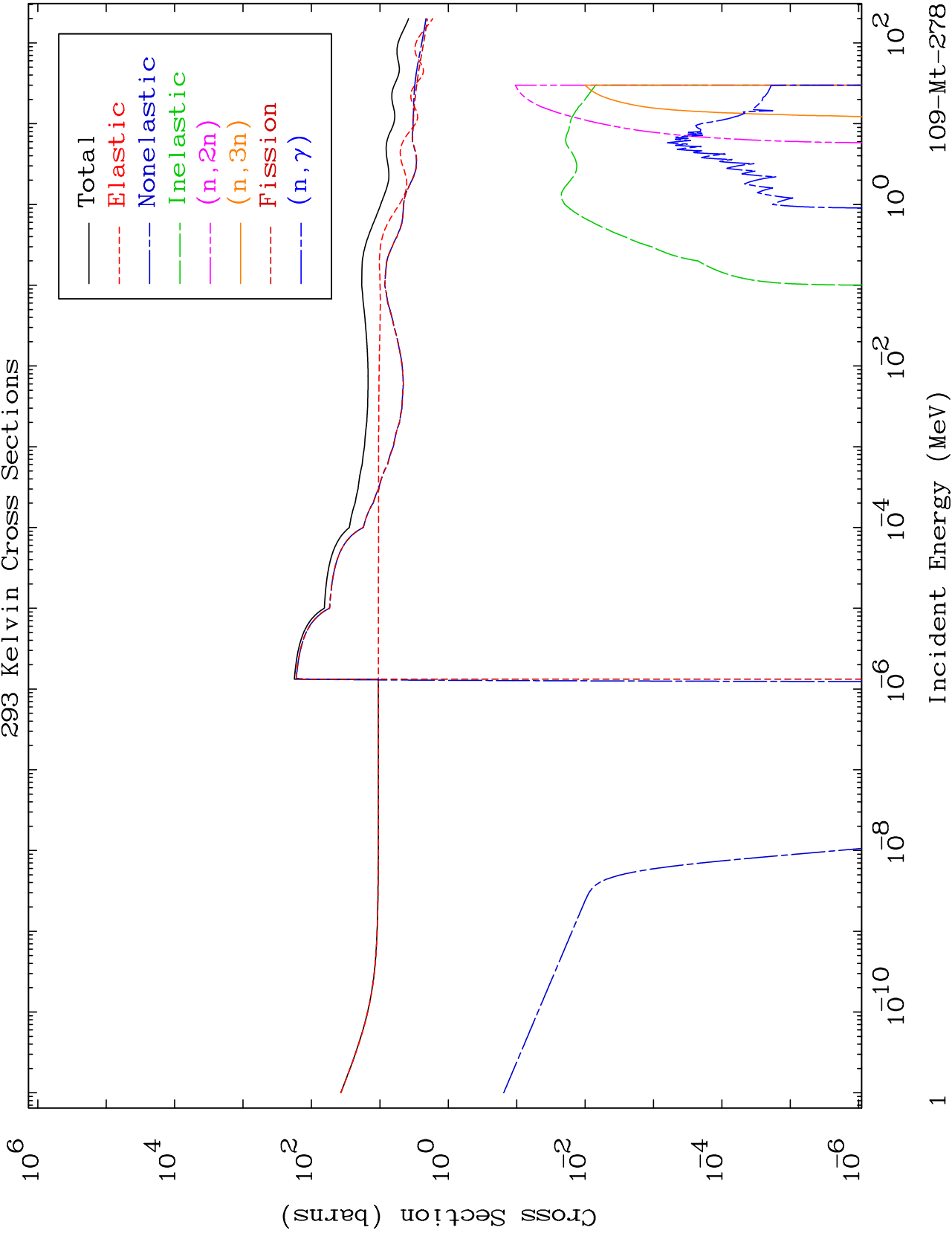
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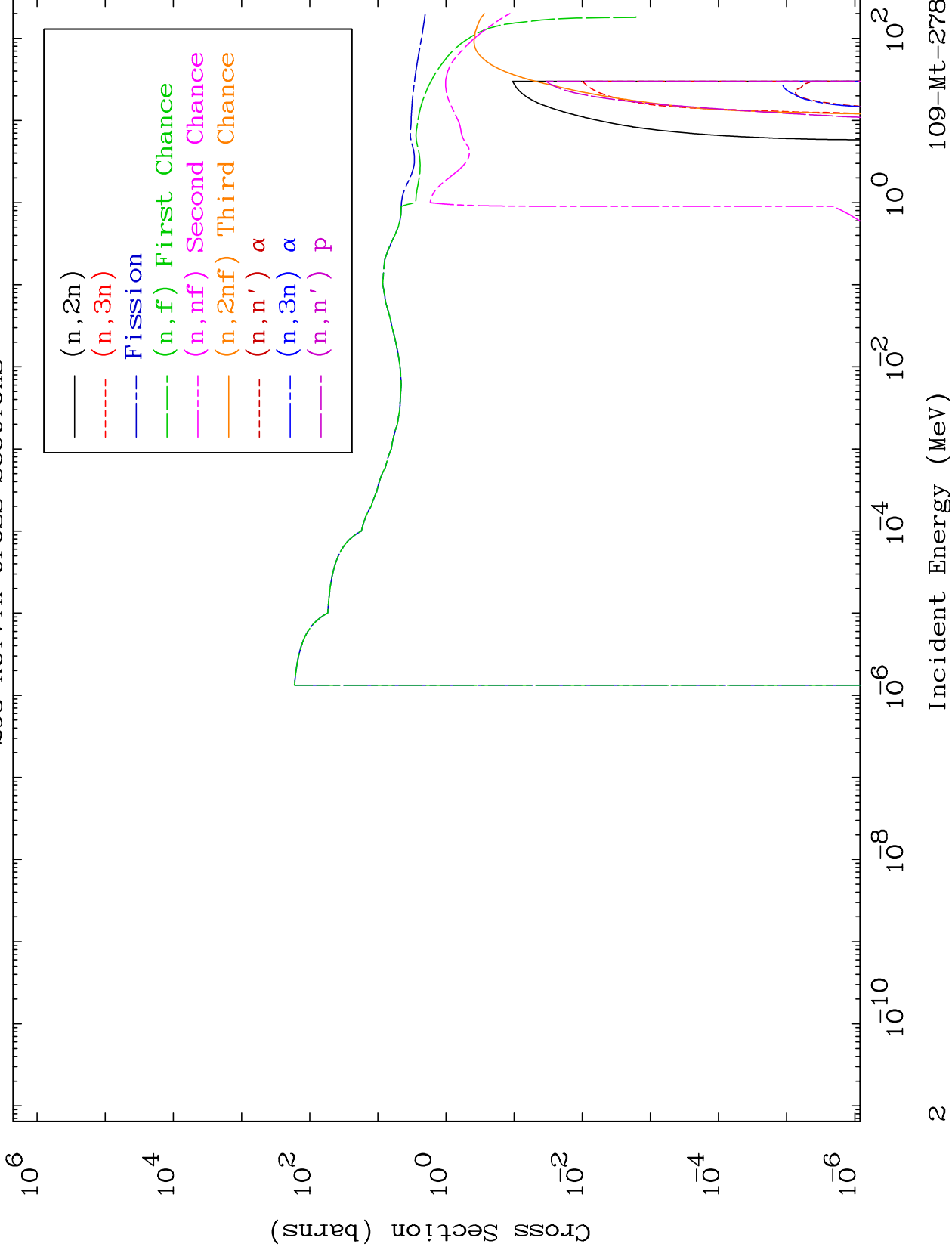
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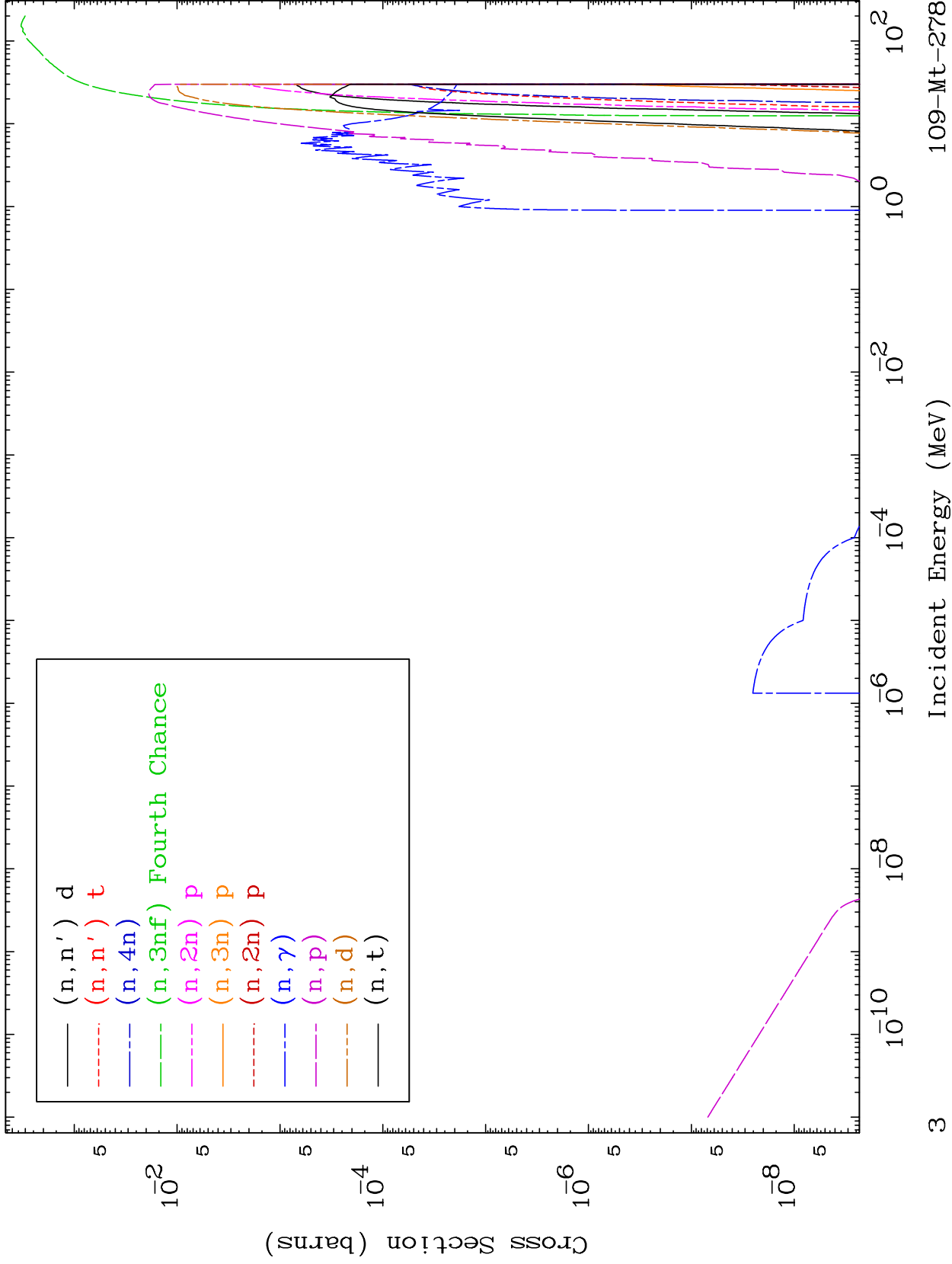
MAT 978

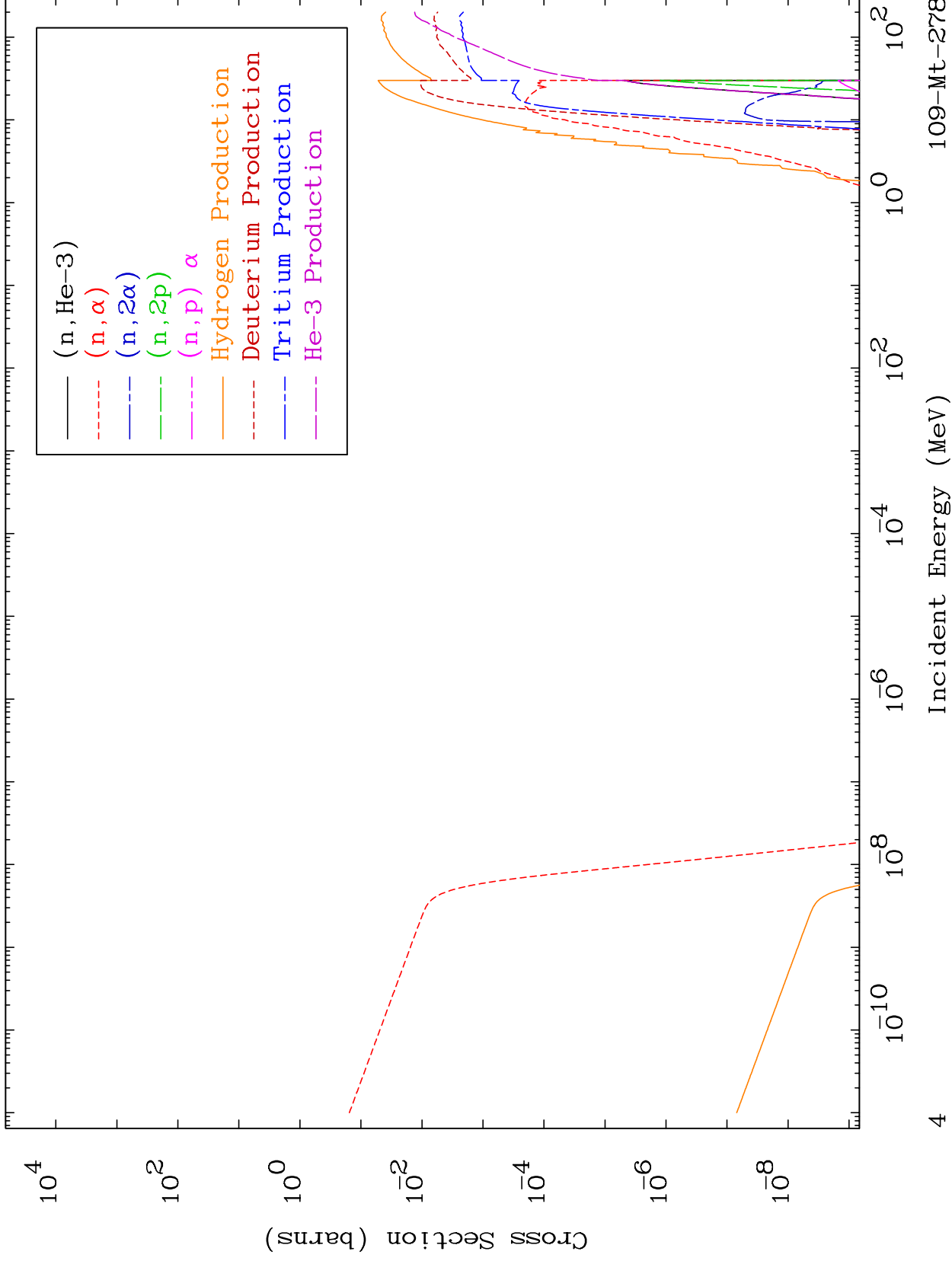
Neutron Major
293 Kelvin Cross Sections

109-Mt-278





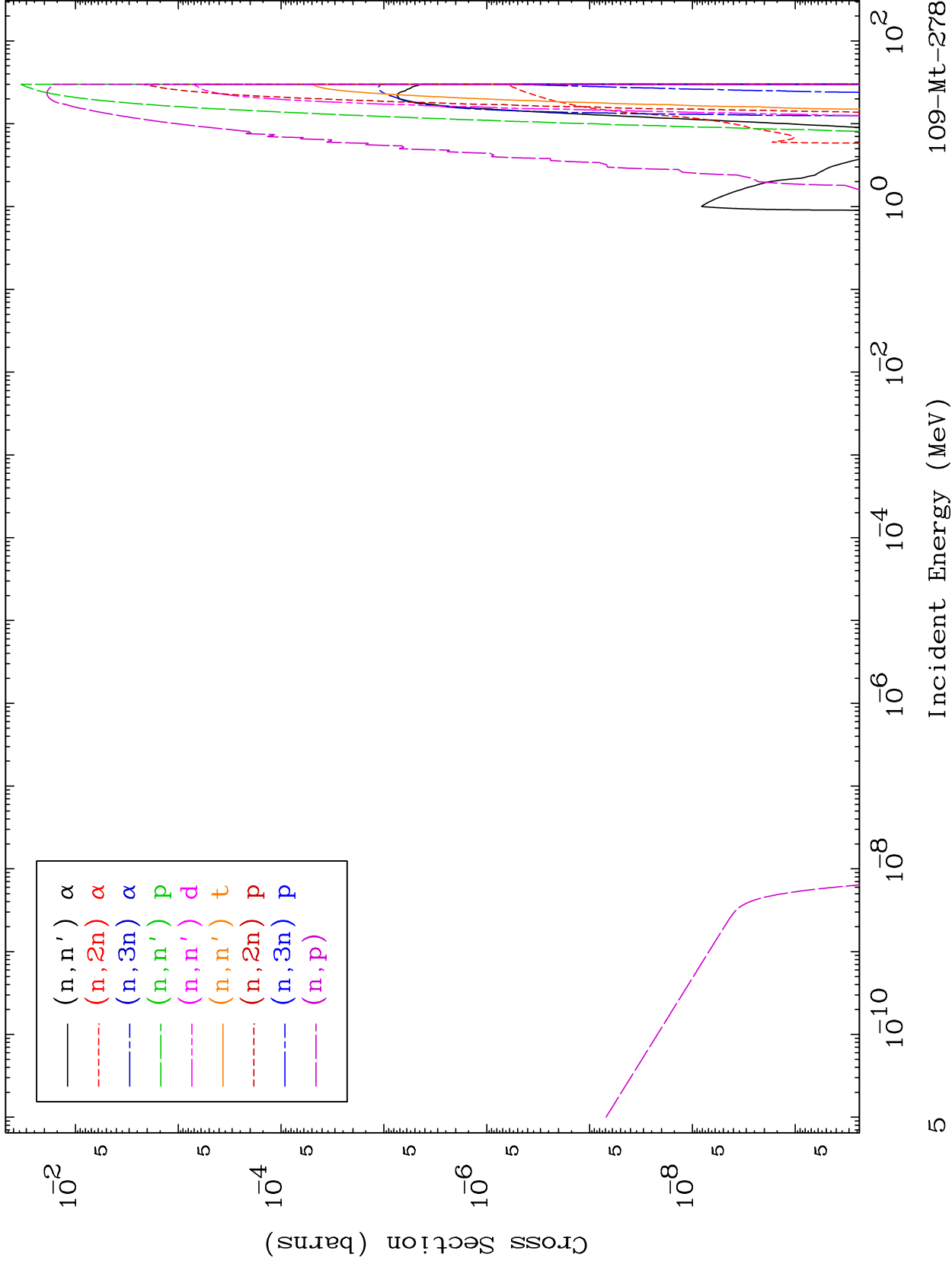




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Charged Particle
293 Kelvin Cross Sections

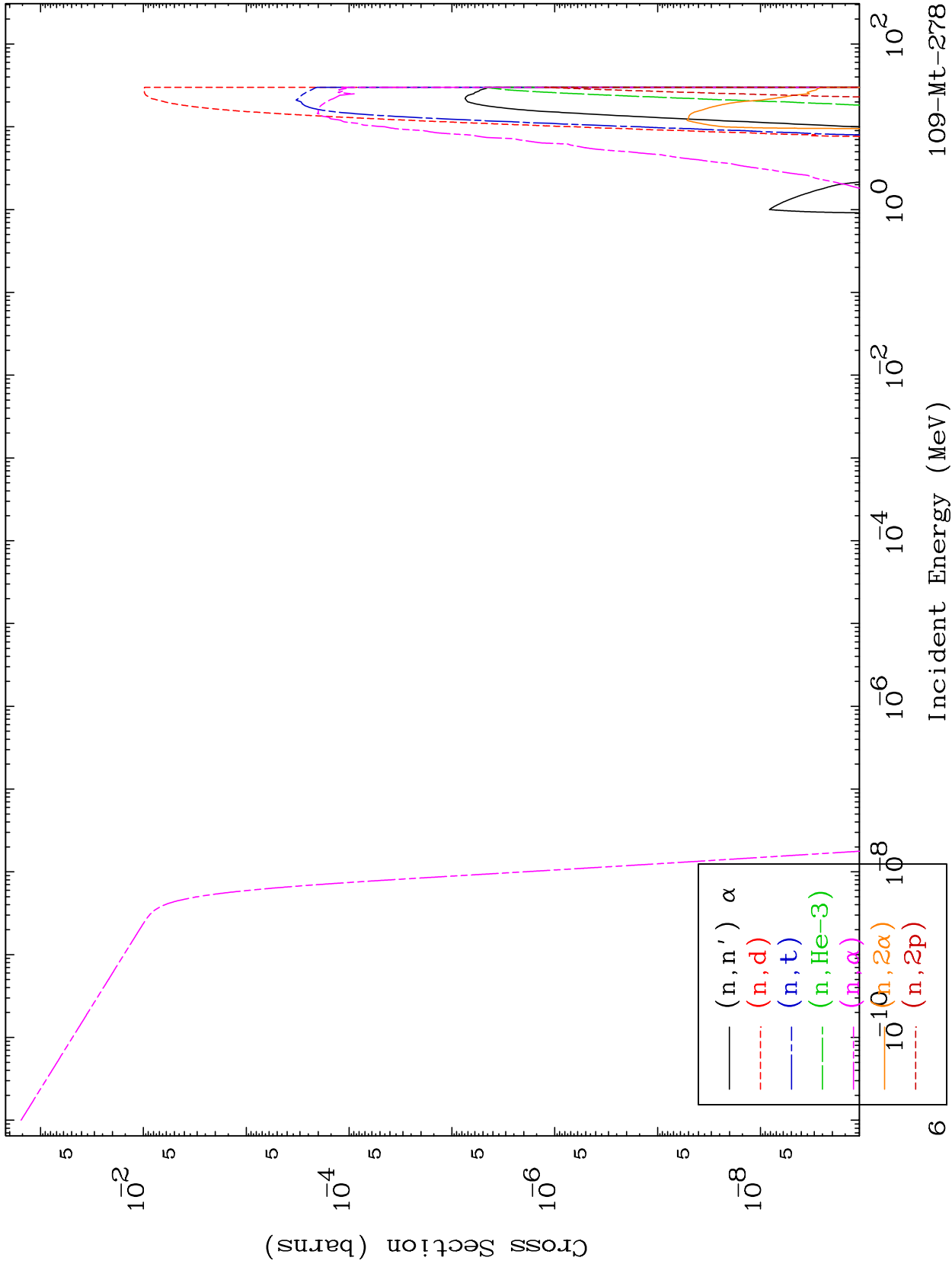
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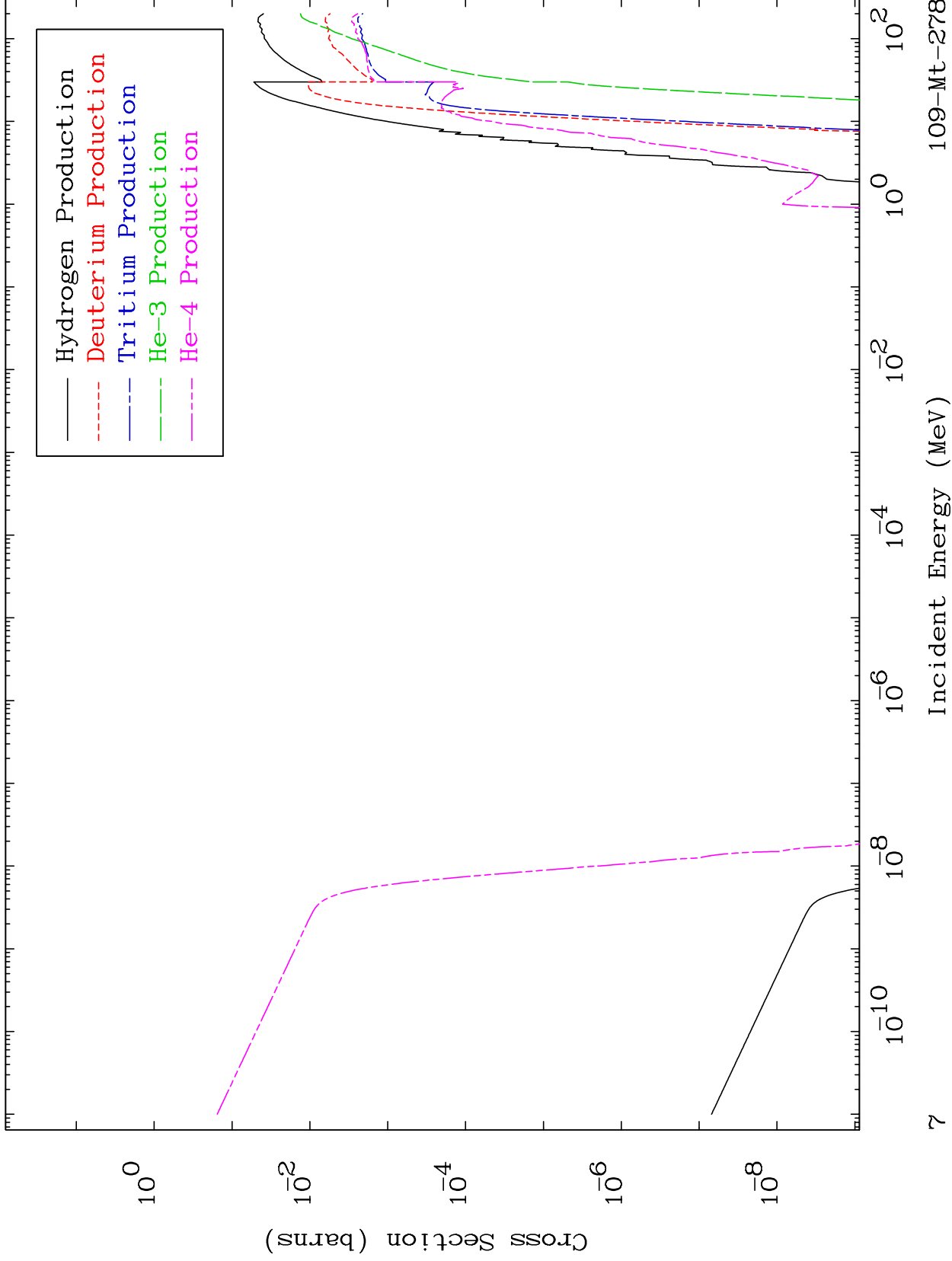
MAT 978

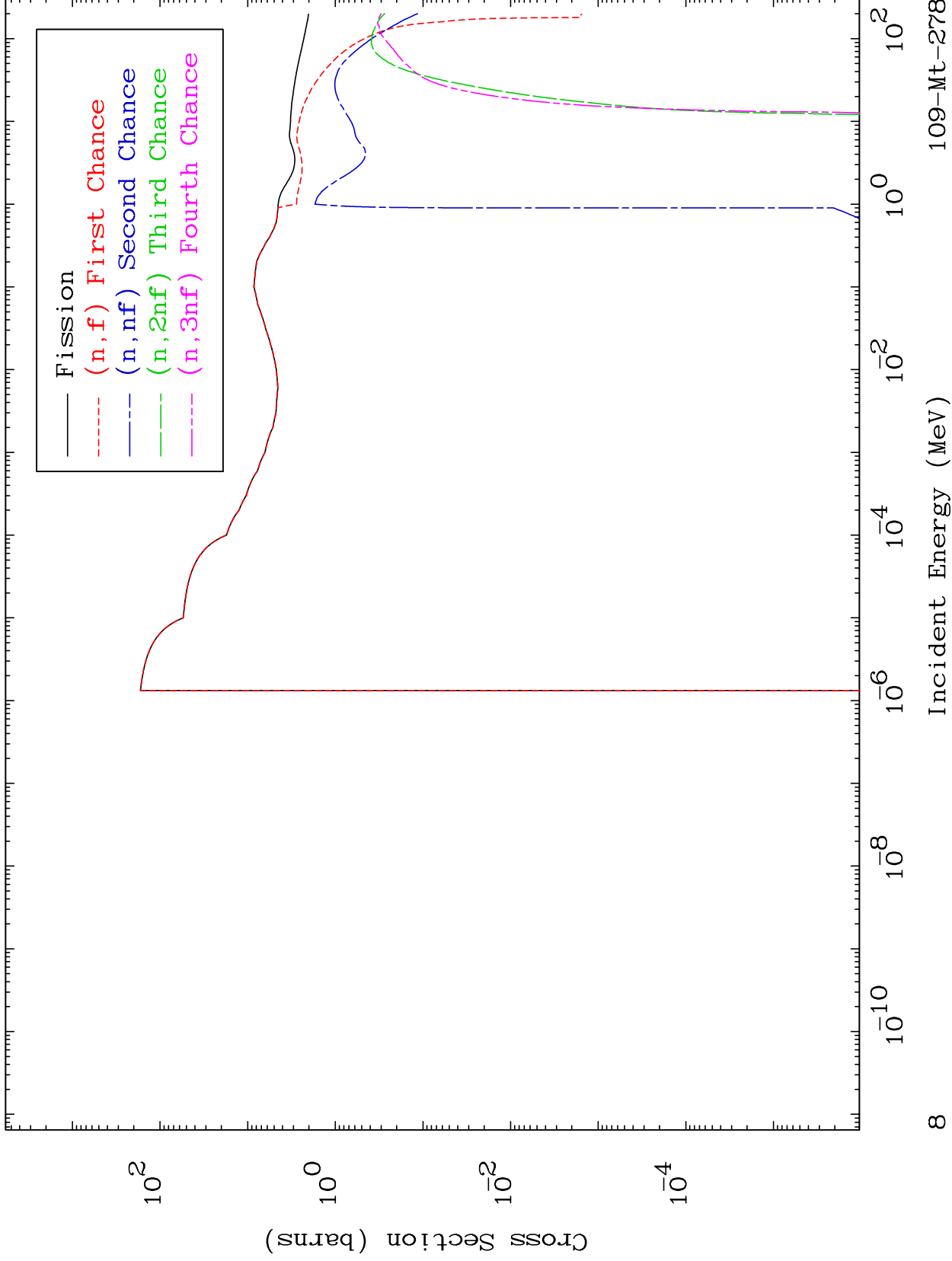
Charged Particle
293 Kelvin Cross Sections

109-Mt-278



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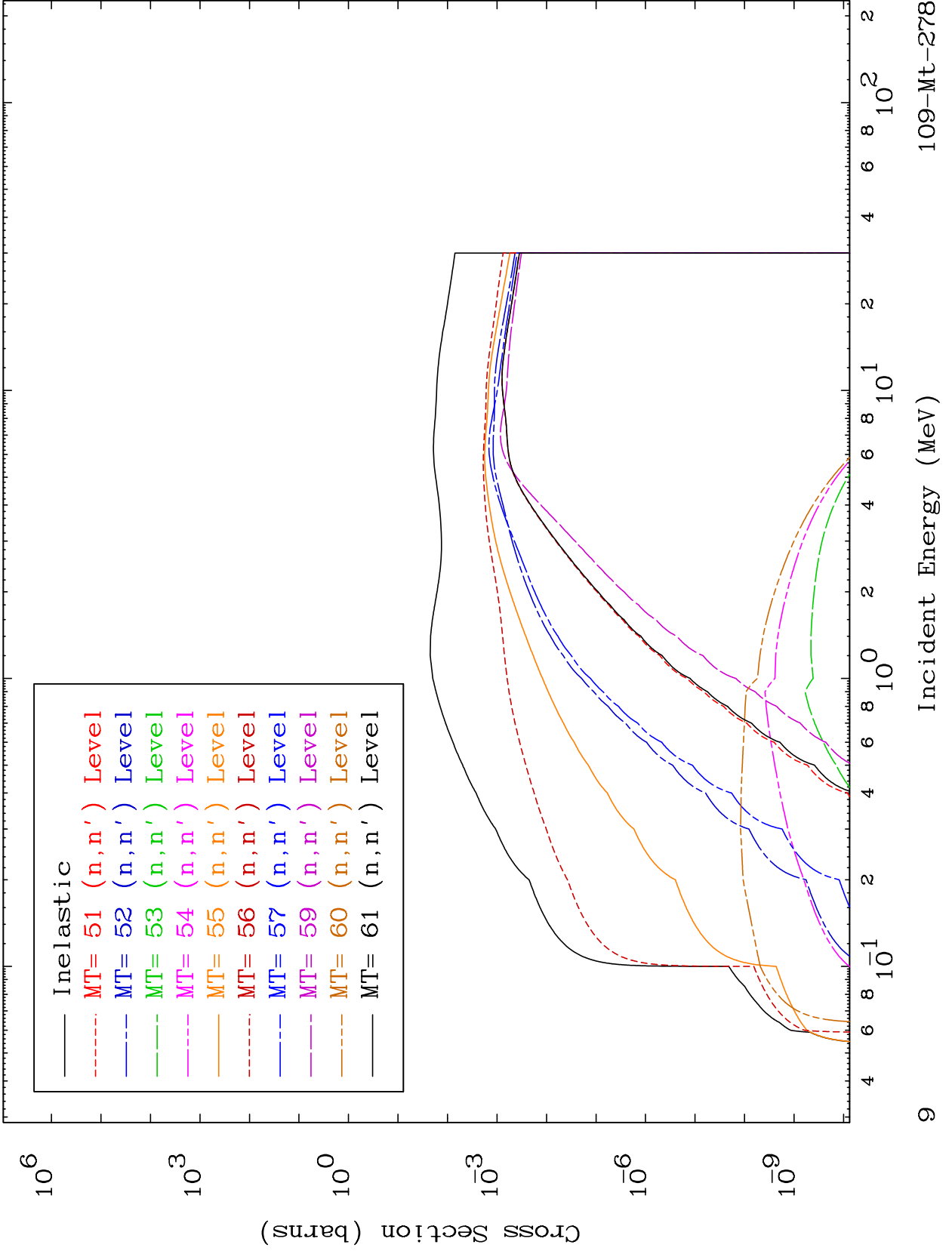




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(n,n') Levels
293 Kelvin Cross Sections

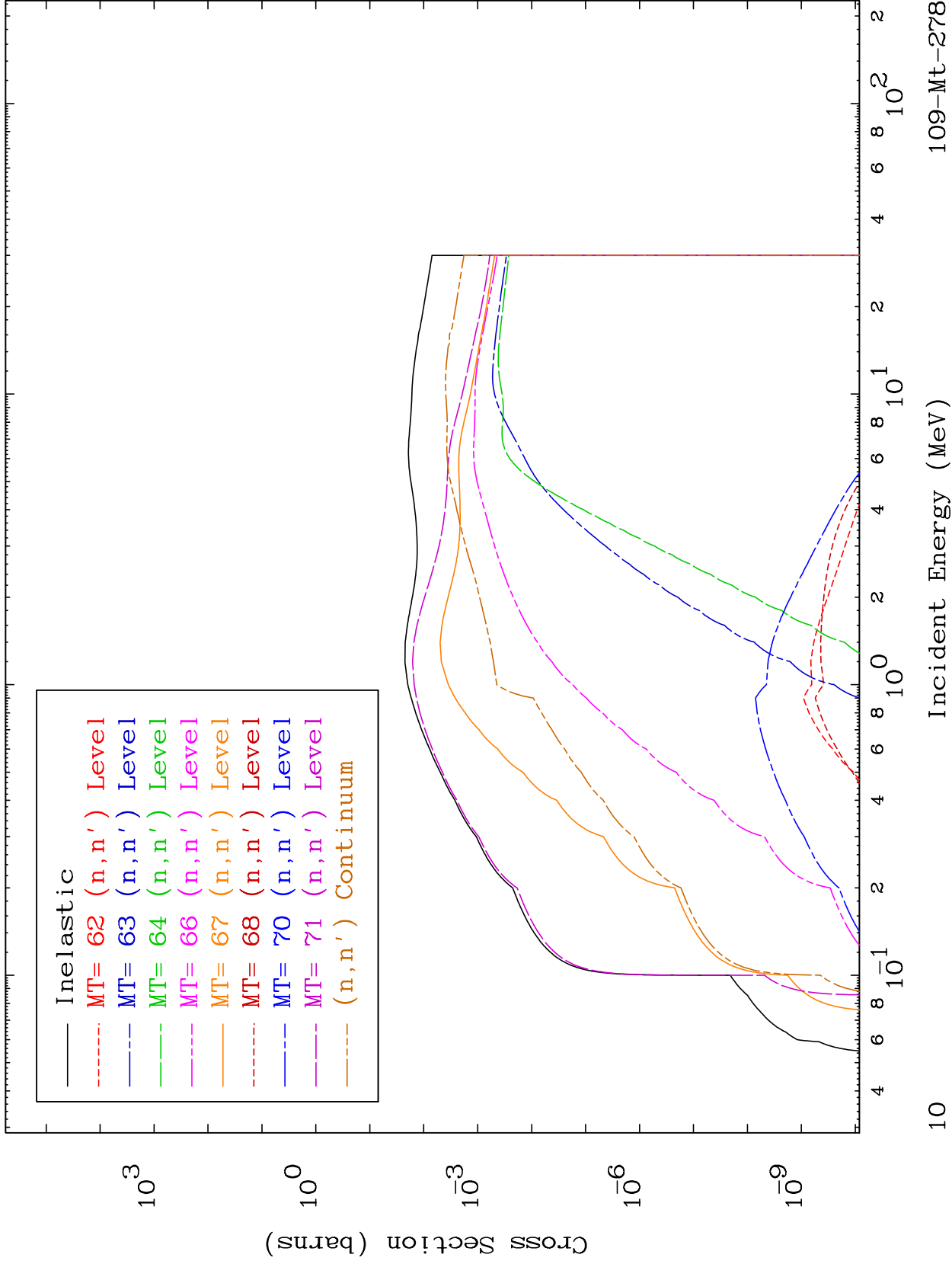
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(n,n') Levels
293 Kelvin Cross Sections

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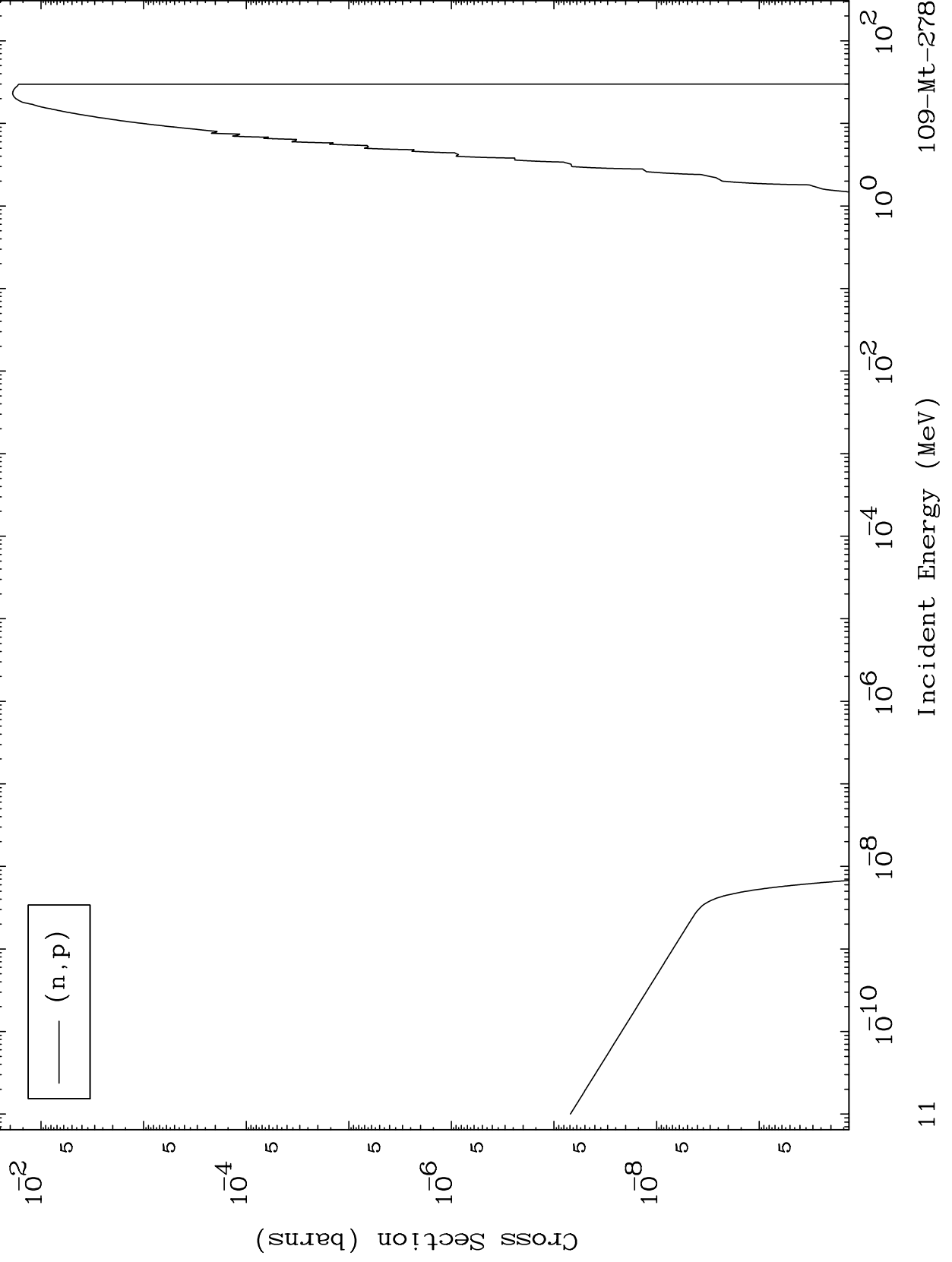
Incident Energy (MeV)

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(n,p) Levels
293 Kelvin Cross Sections

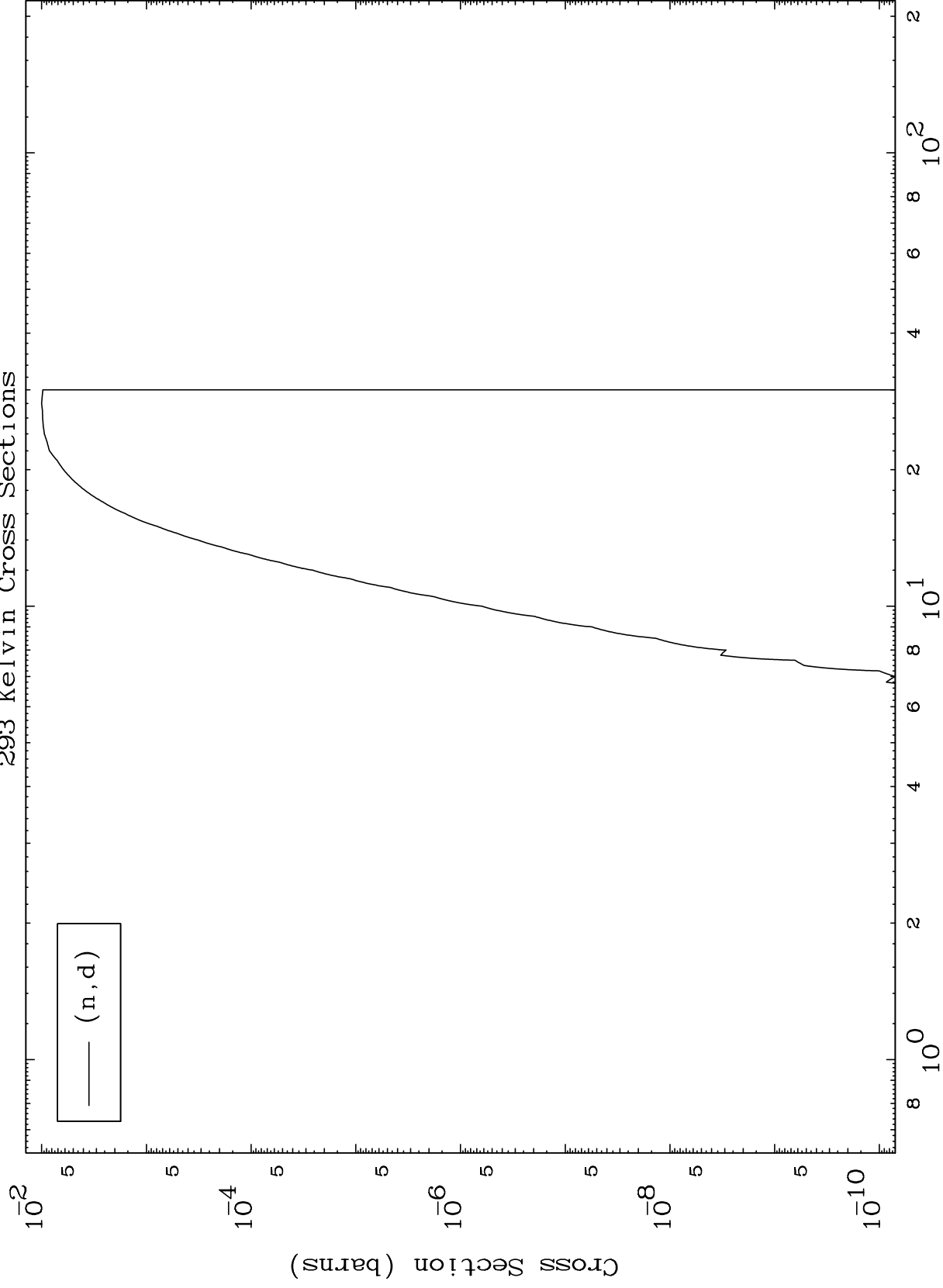
109-Mt-278



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(n,d) Levels
293 Kelvin Cross Sections

109-Mt-278



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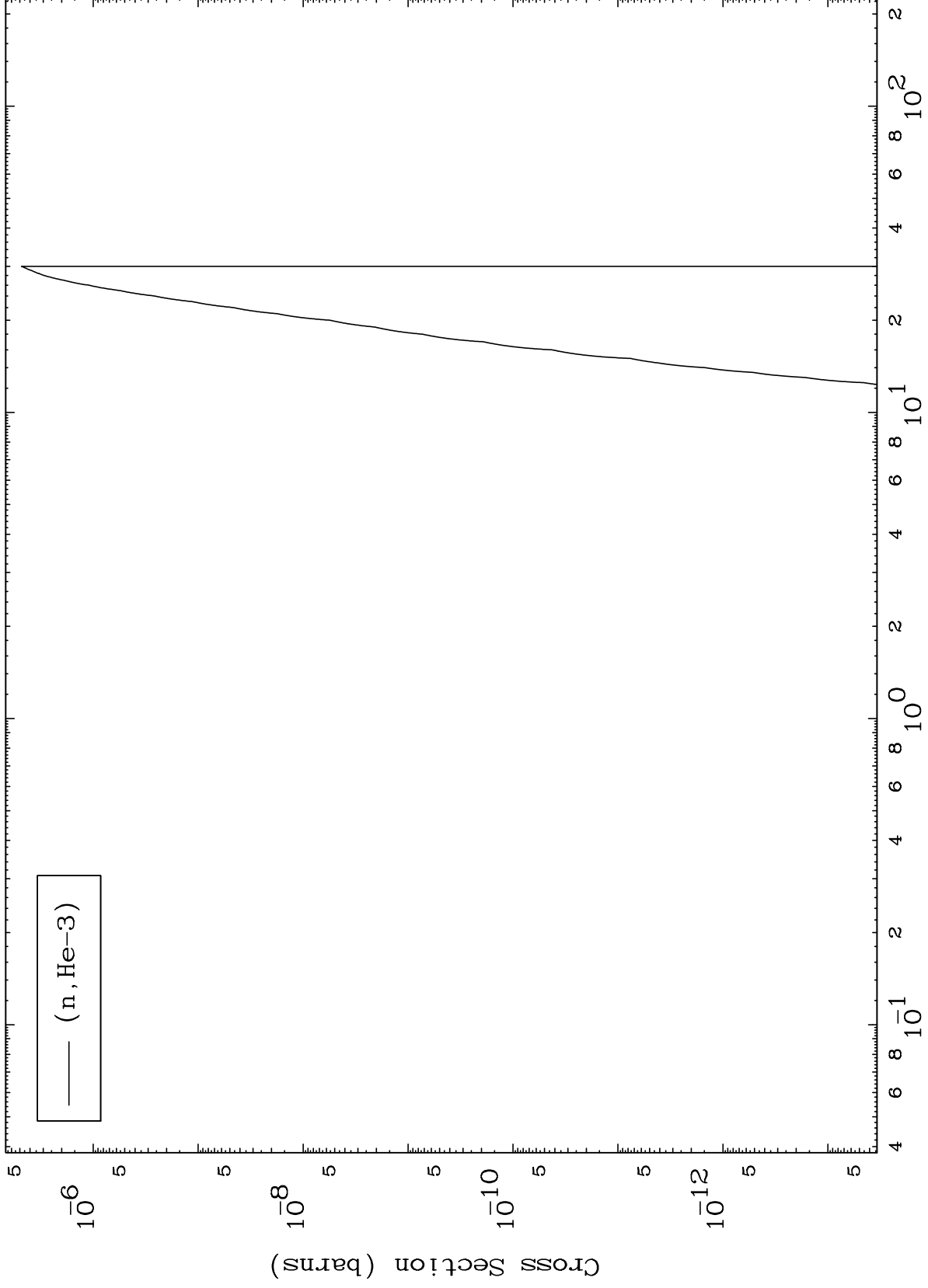
Incident Energy (MeV)

109-Mt-278

MAT 978

(n,He3) Levels
293 Kelvin Cross Sections

109-Mt-278



14

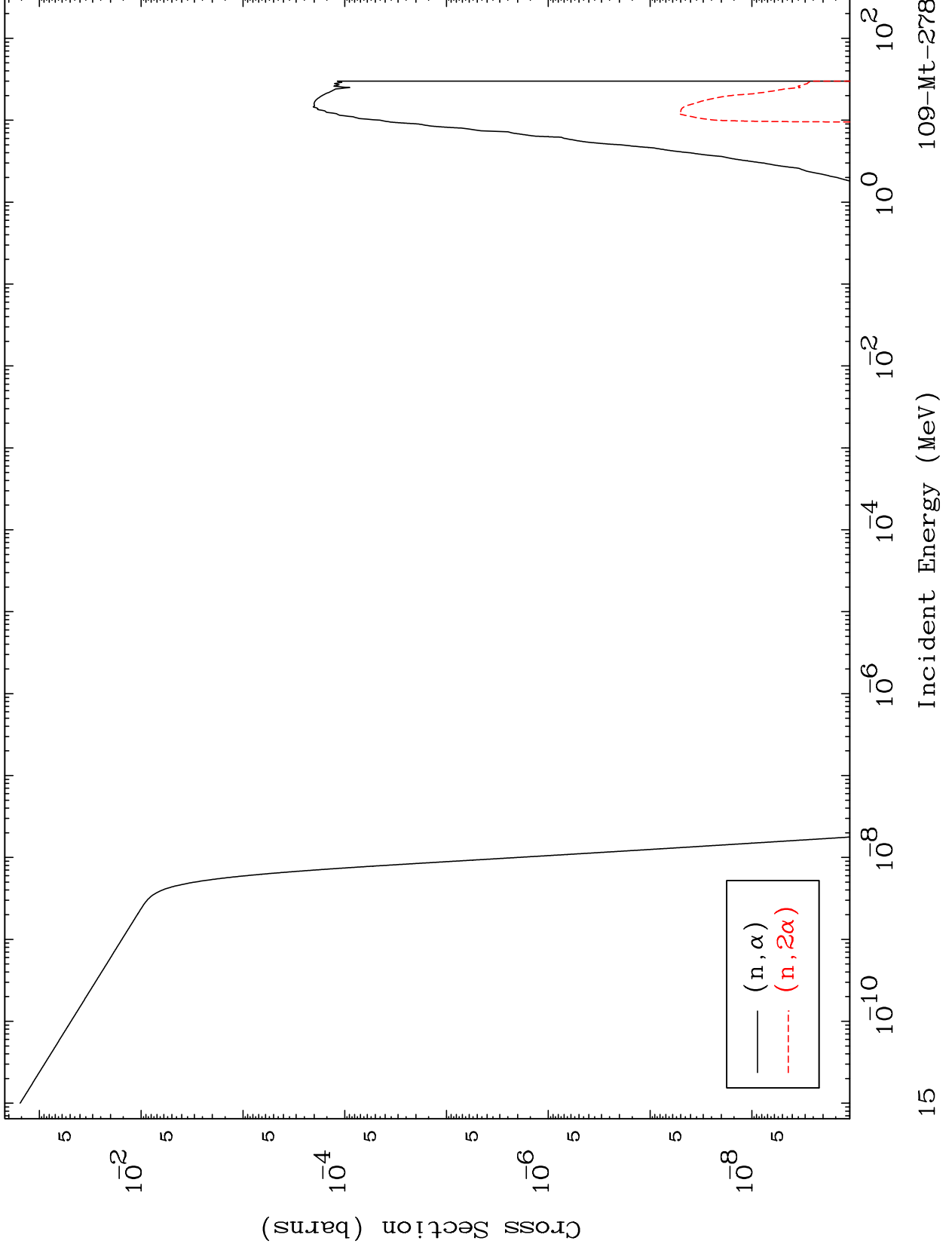
Incident Energy (MeV)

109-Mt-278

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(n, α) Levels
293 Kelvin Cross Sections

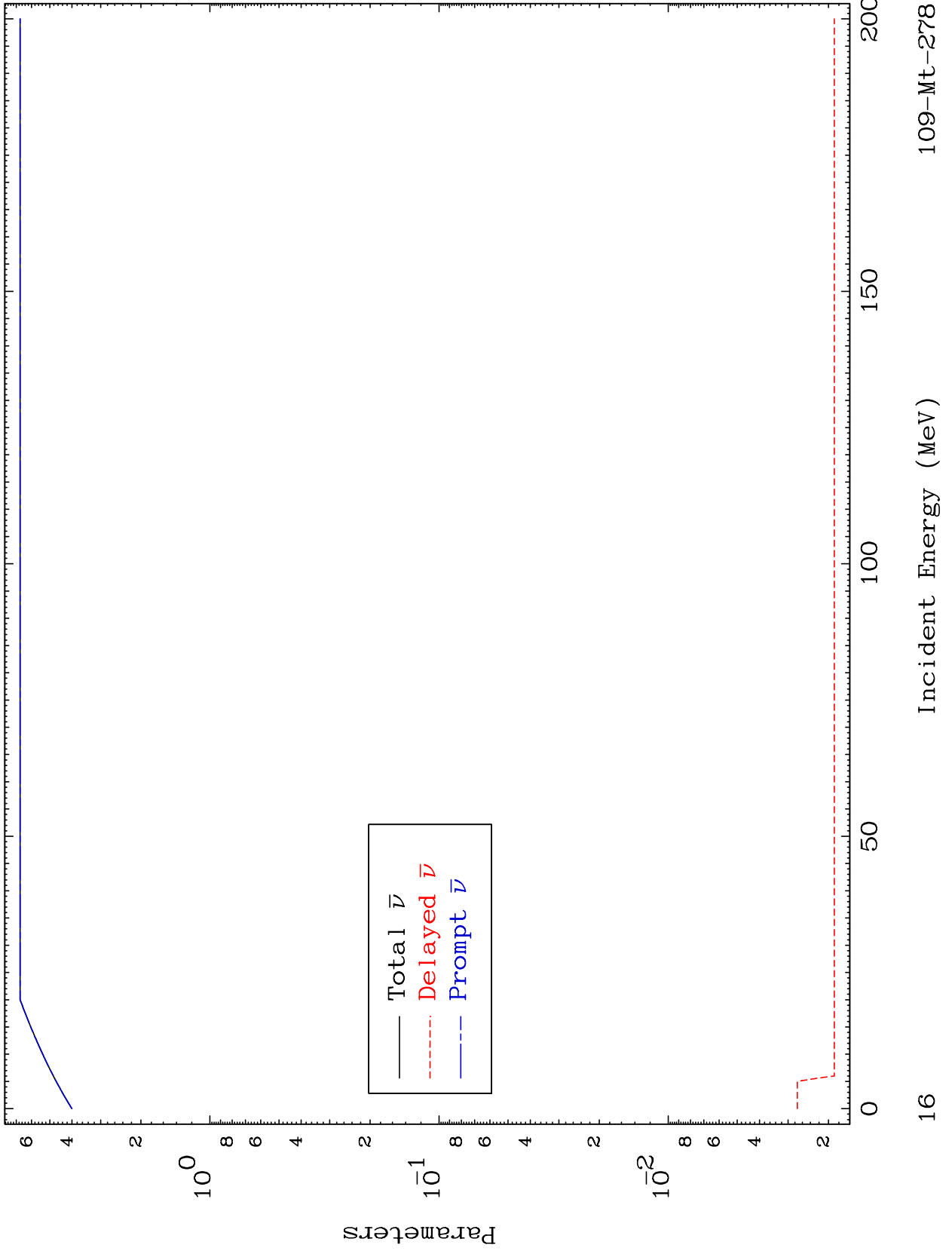
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Energy Release
Parameters

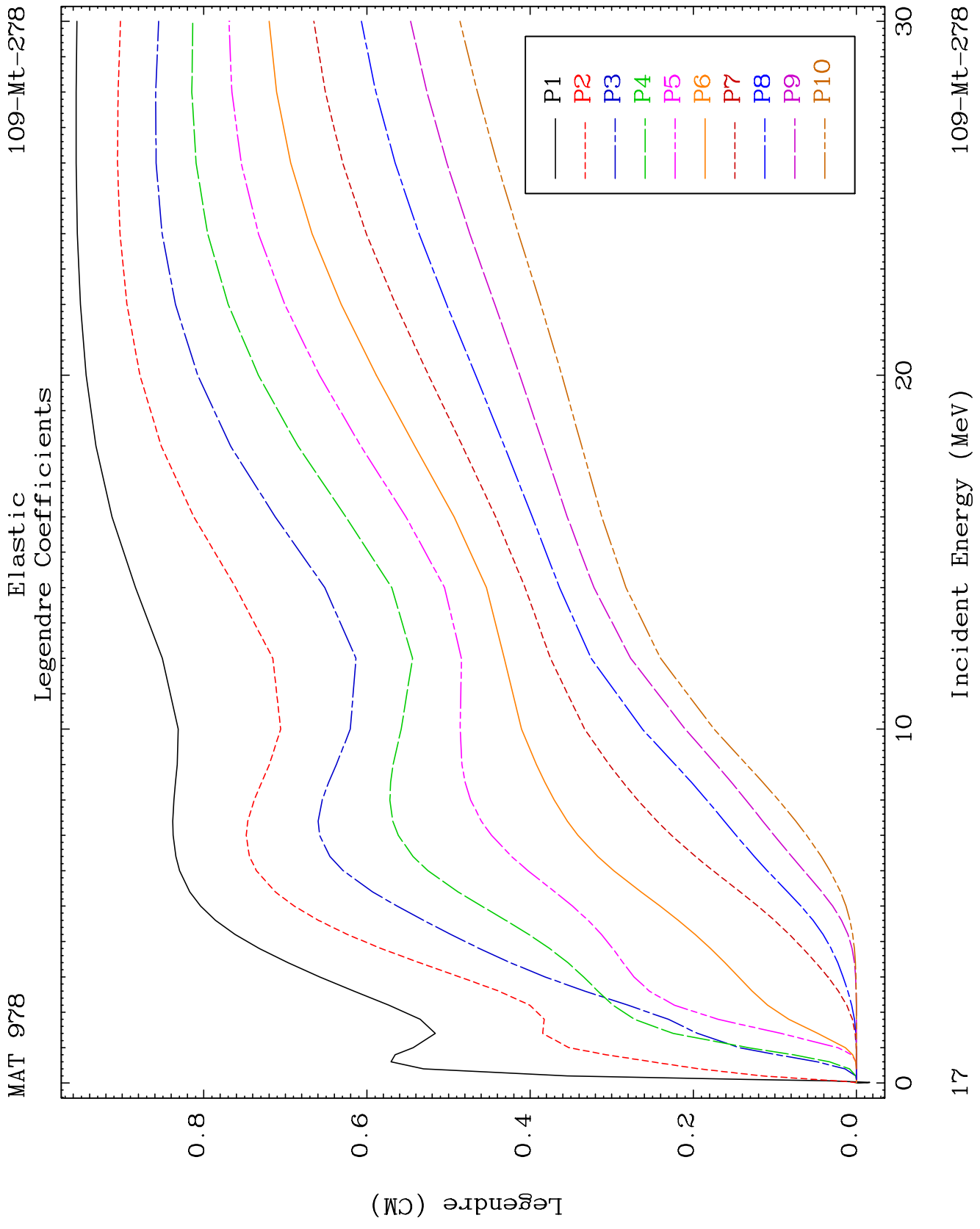
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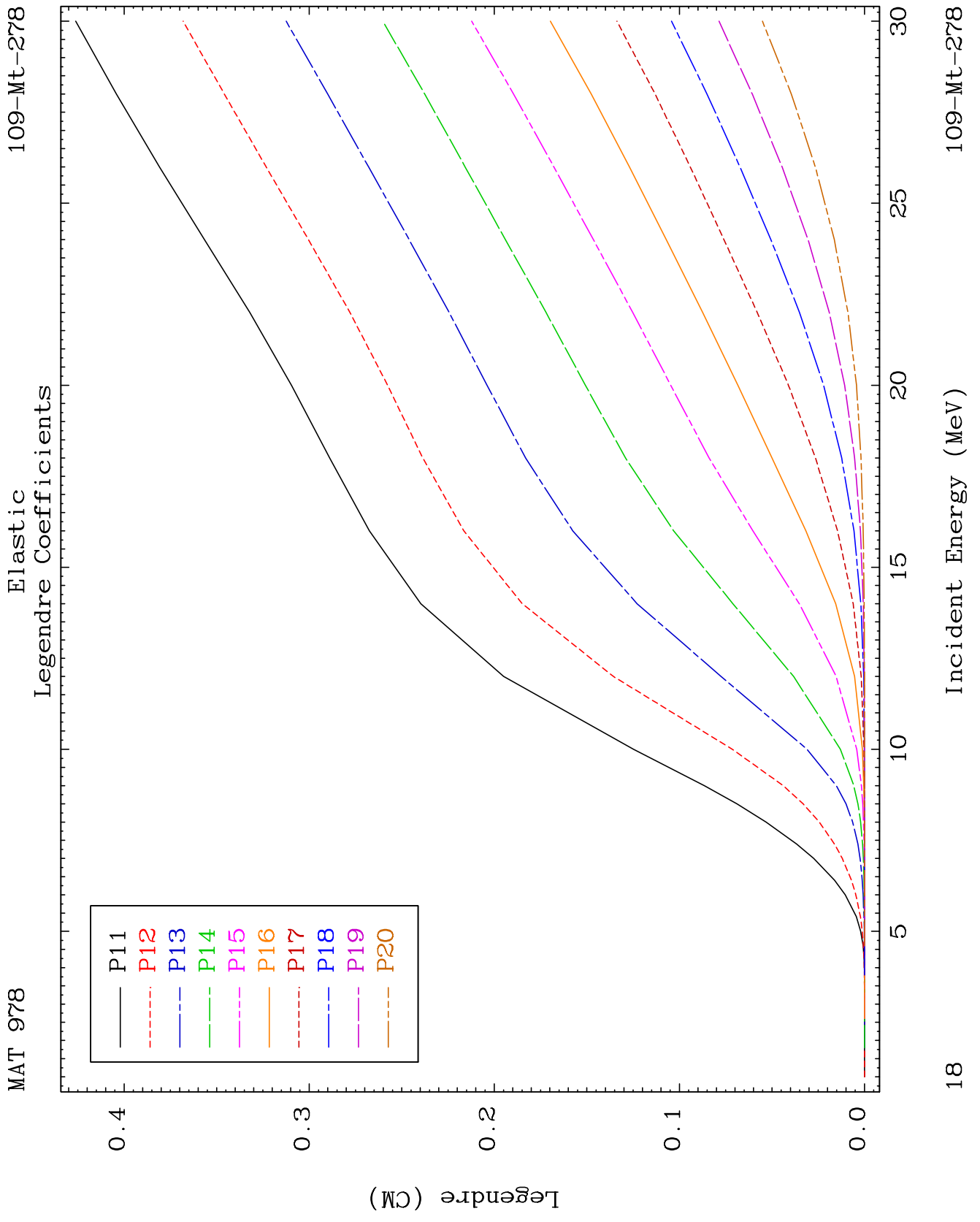


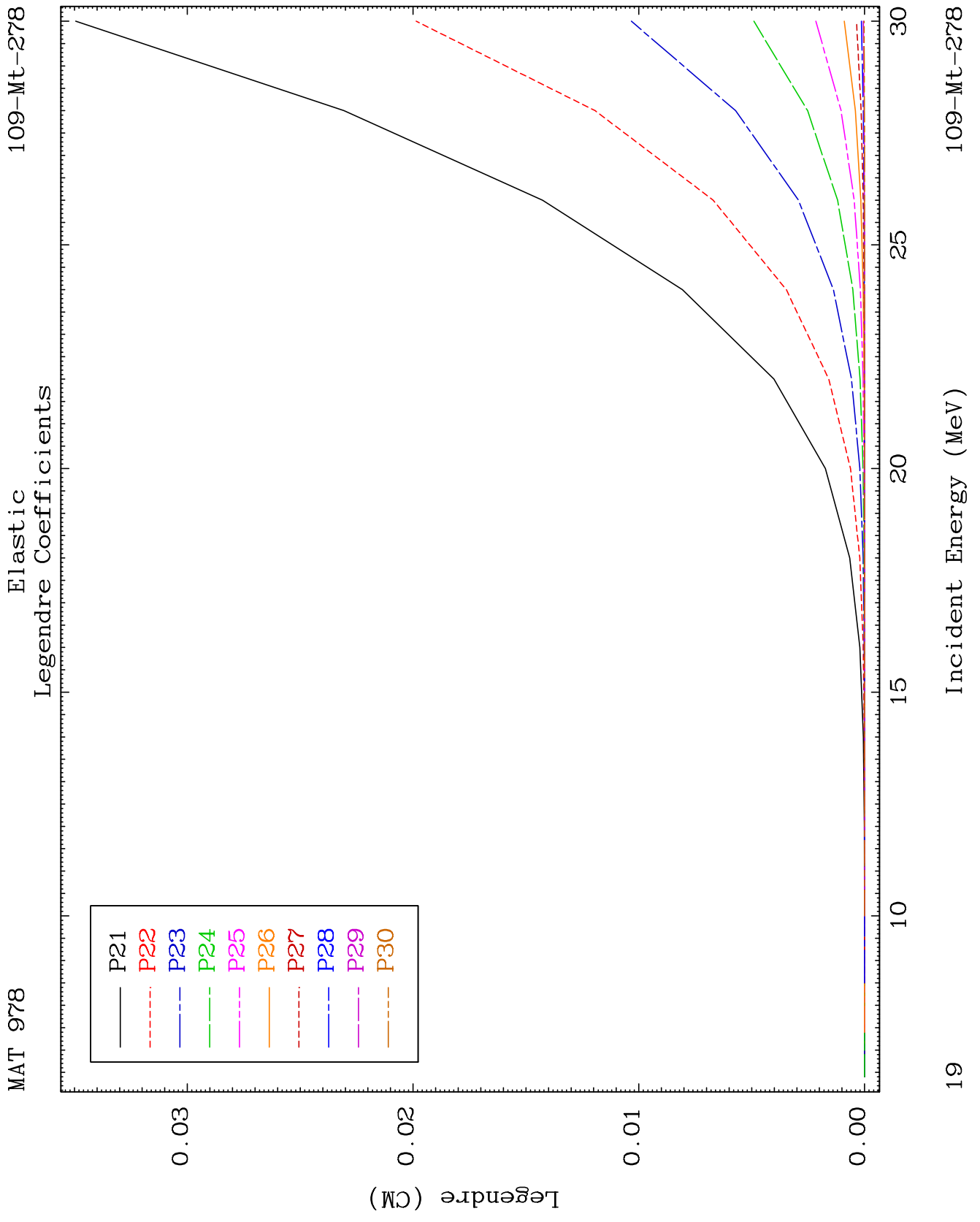
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Incident Energy (MeV)

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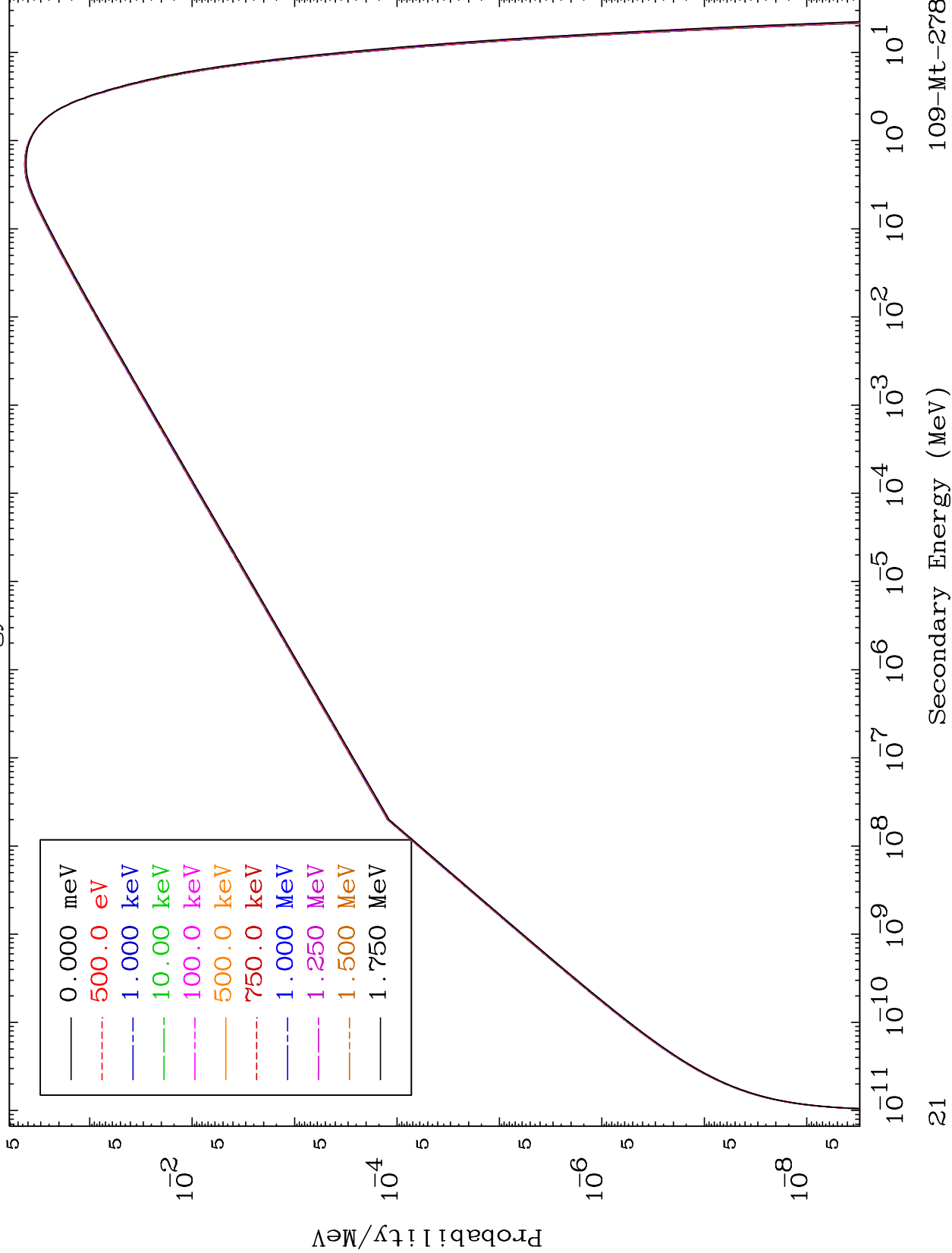




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Fission Energy Distributions

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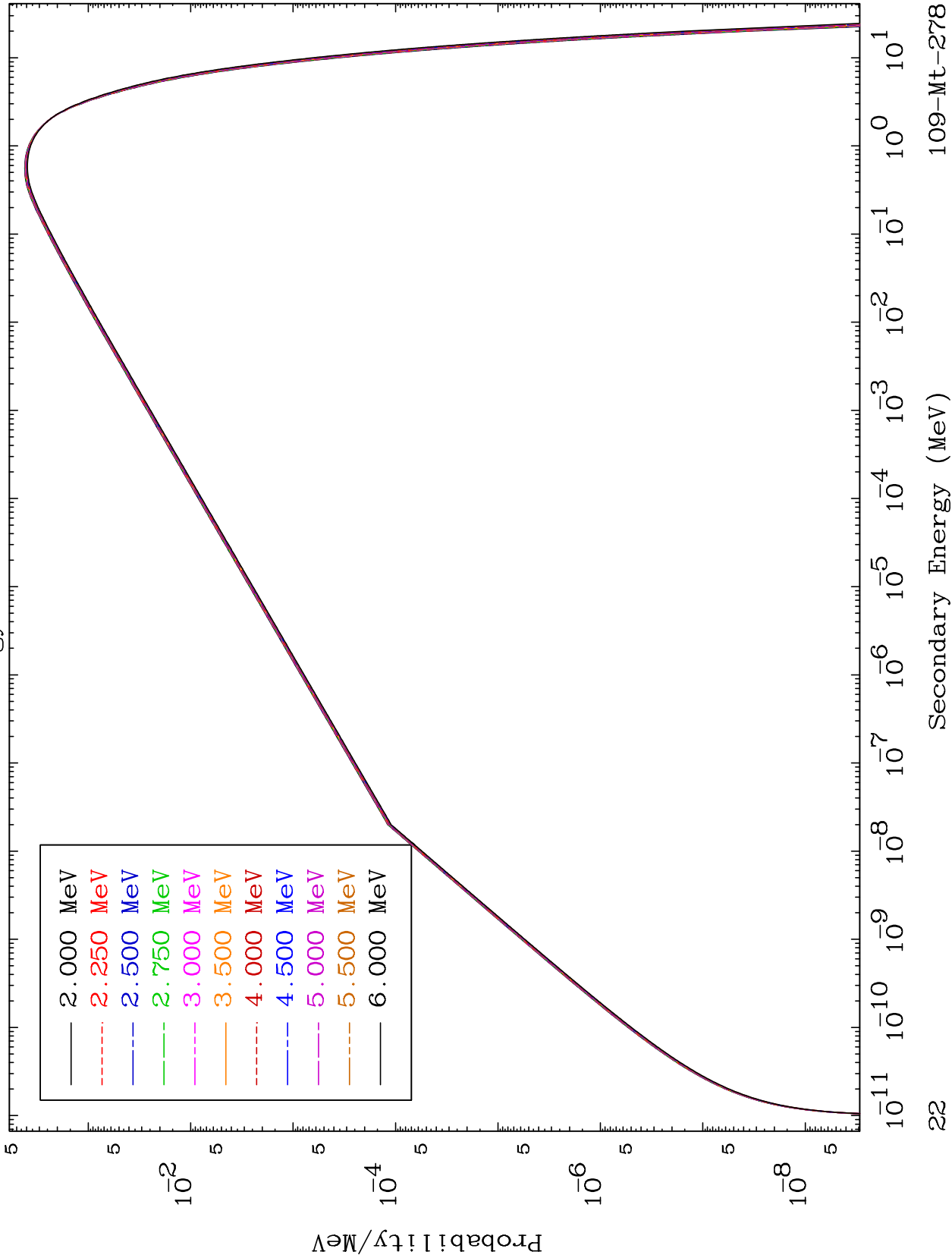
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Secondary Energy (MeV)

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Fission Energy Distributions

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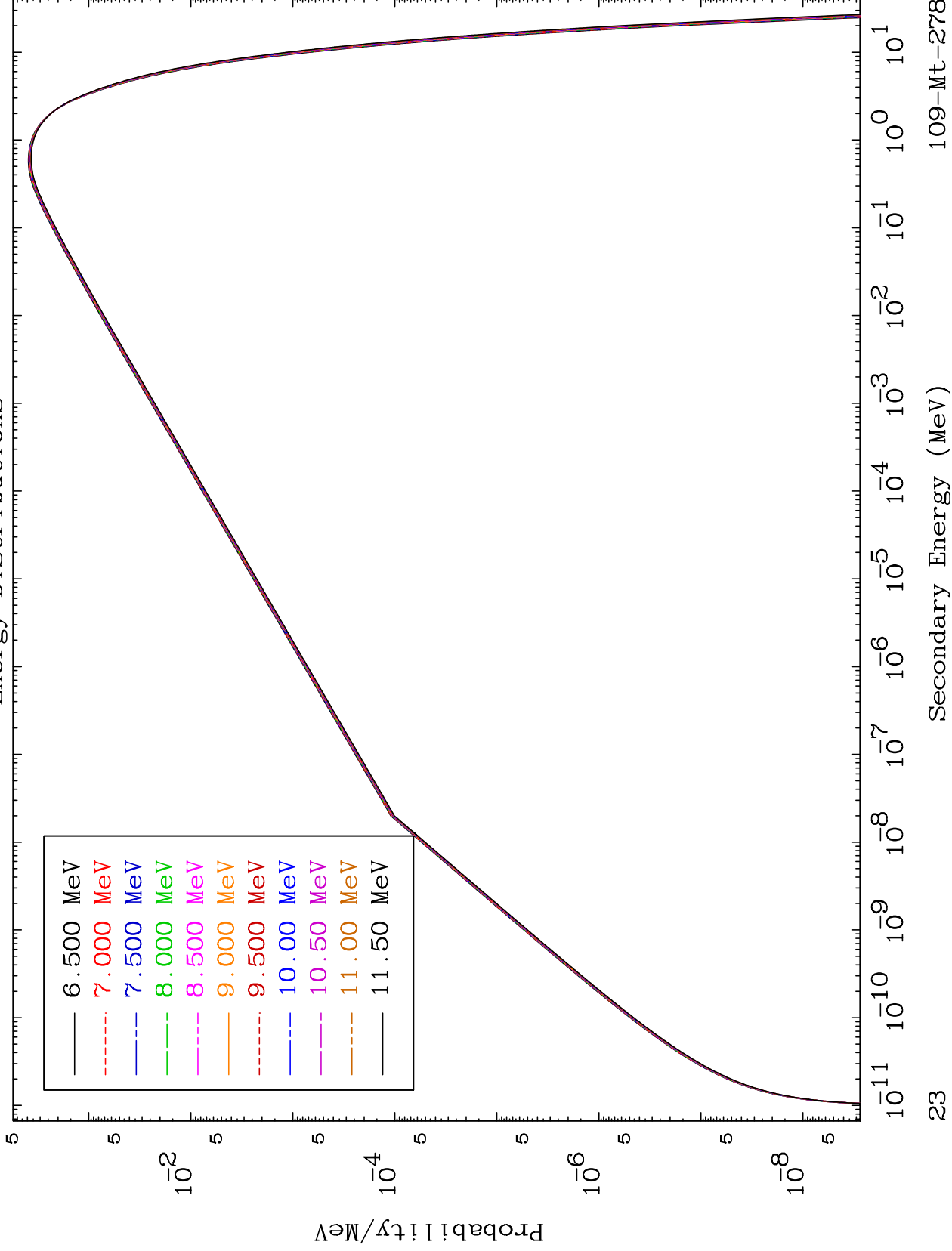


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Fission Energy Distributions

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109-Mt-278

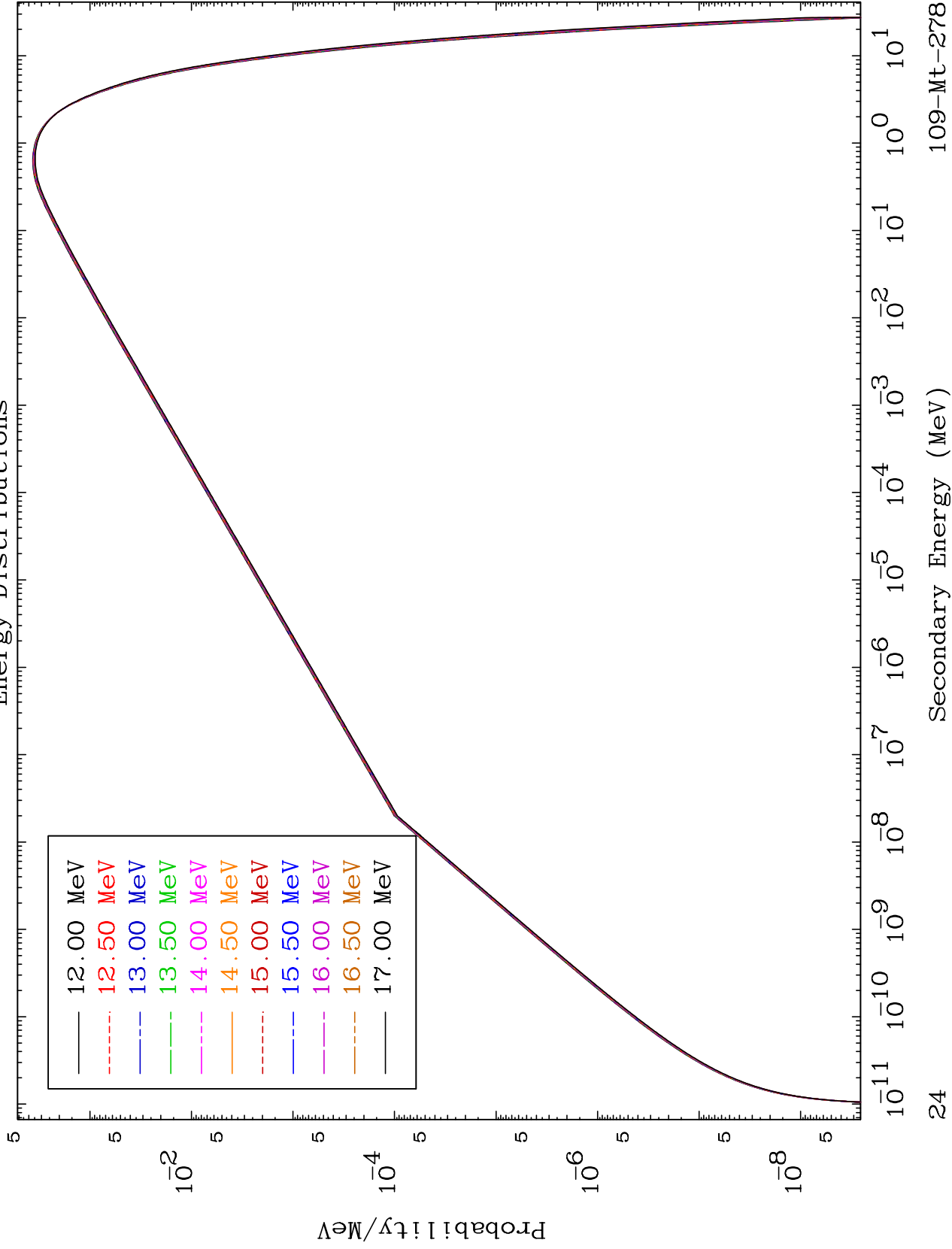
Secondary Energy (MeV)

23

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Fission Energy Distributions

109-Mt-278



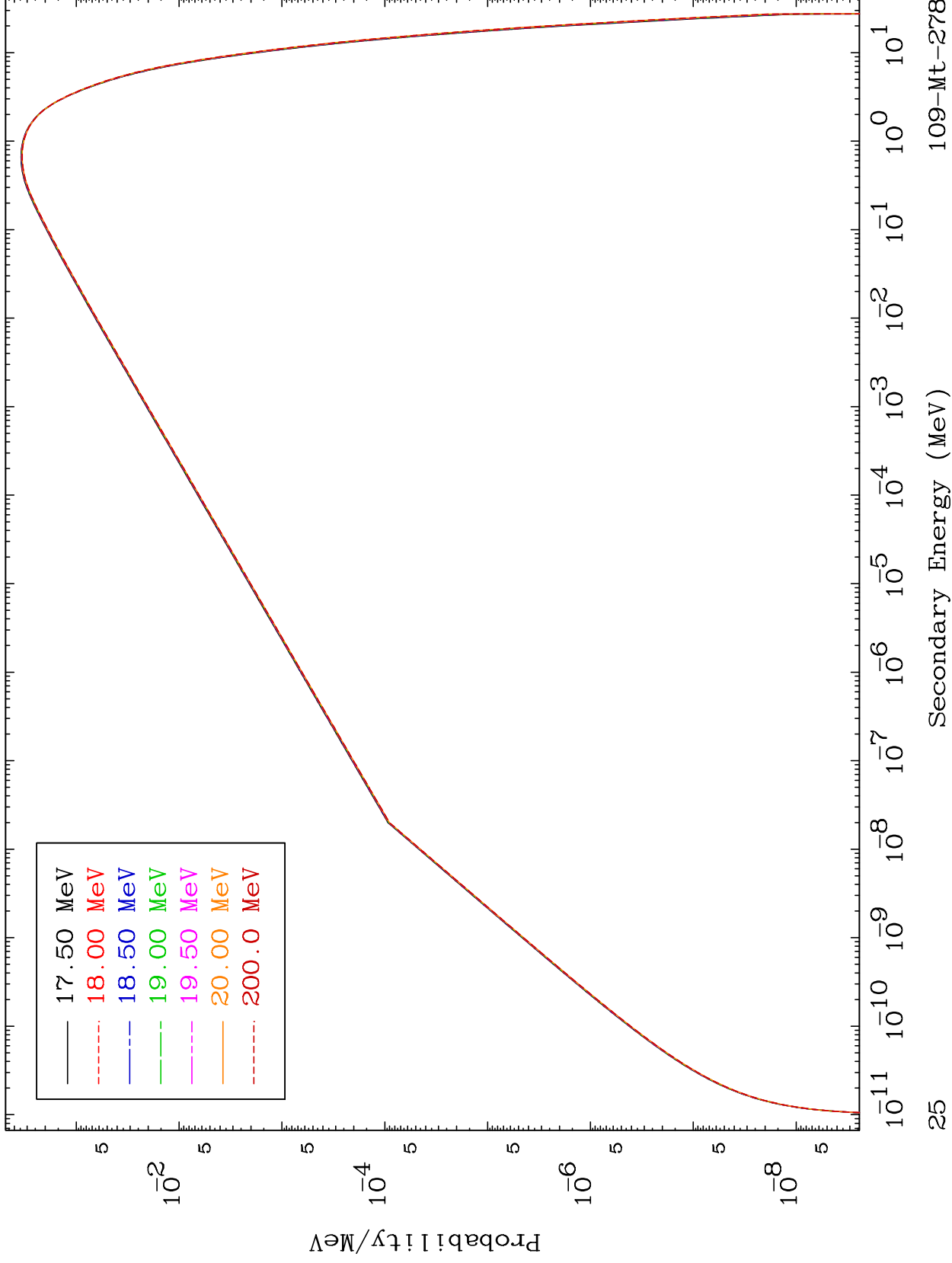
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Fission Energy Distributions

109-Mt-278



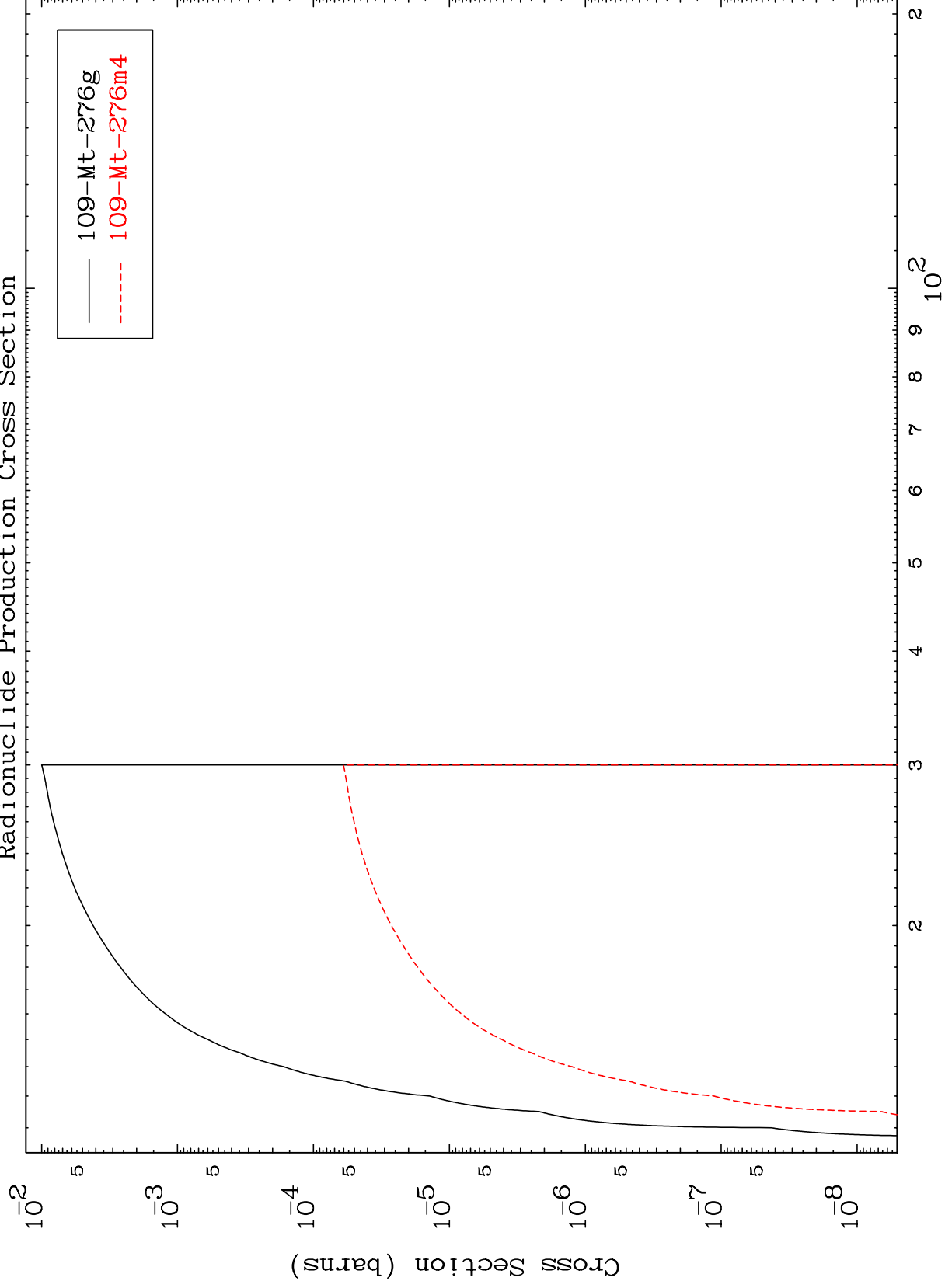
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(n,3n)

Radionuclide Production Cross Section



26

Incident Energy (MeV)

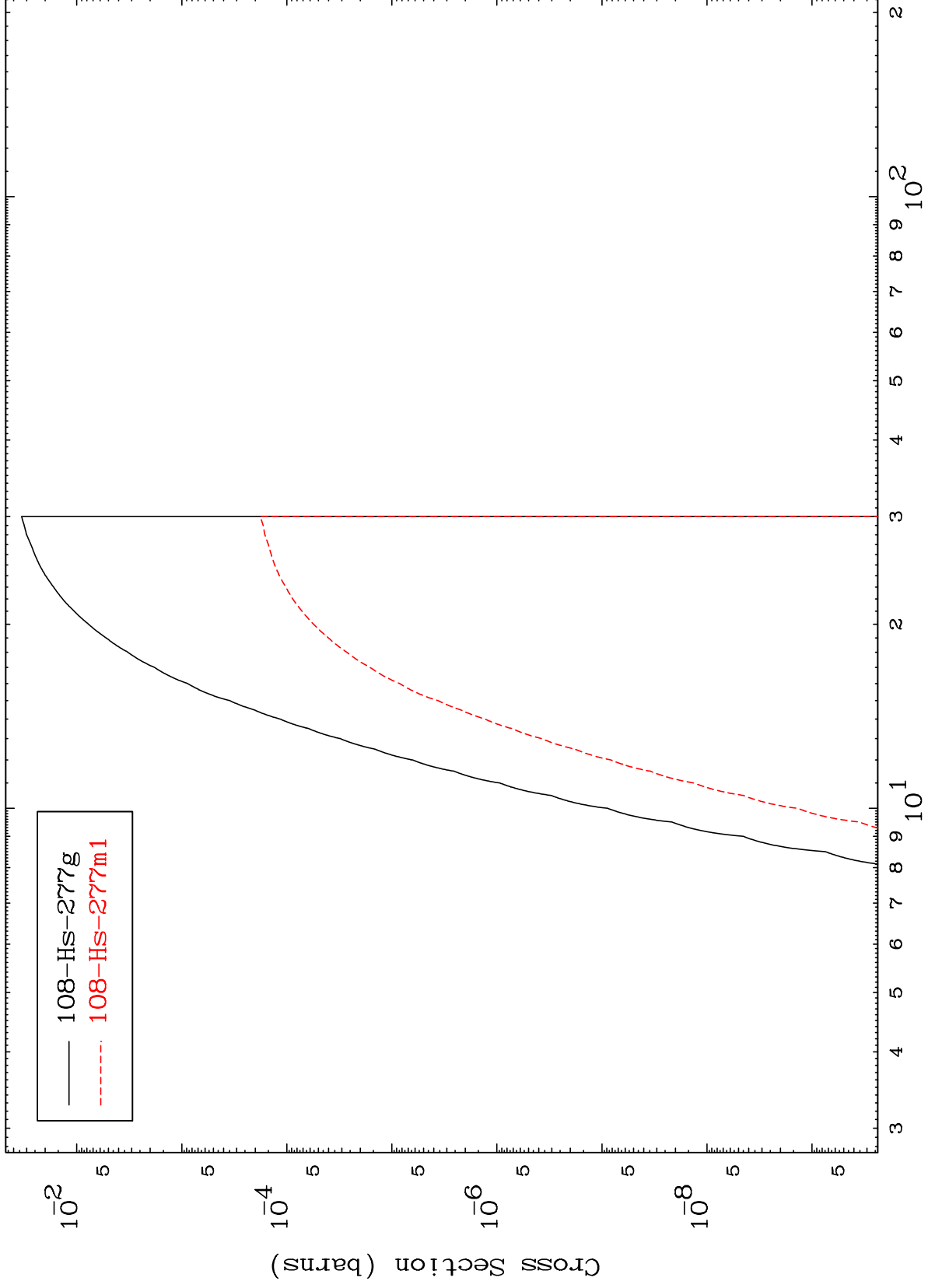
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(n,n') p

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Radionuclide Production Cross Section



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