

Program EVALPLOT  
(Version 2021-1)

by

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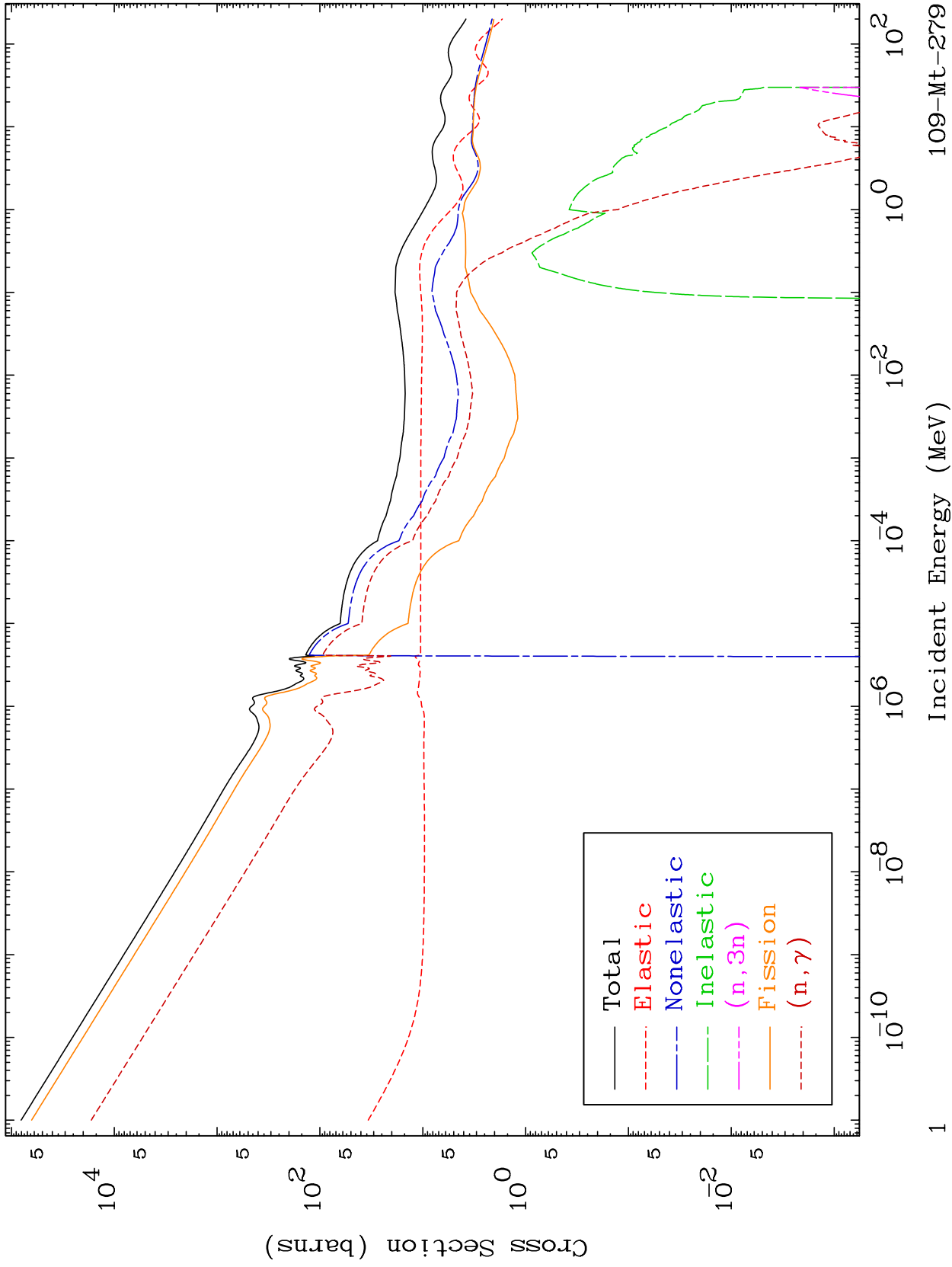
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 979

Neutron Major  
293 Kelvin Cross Sections

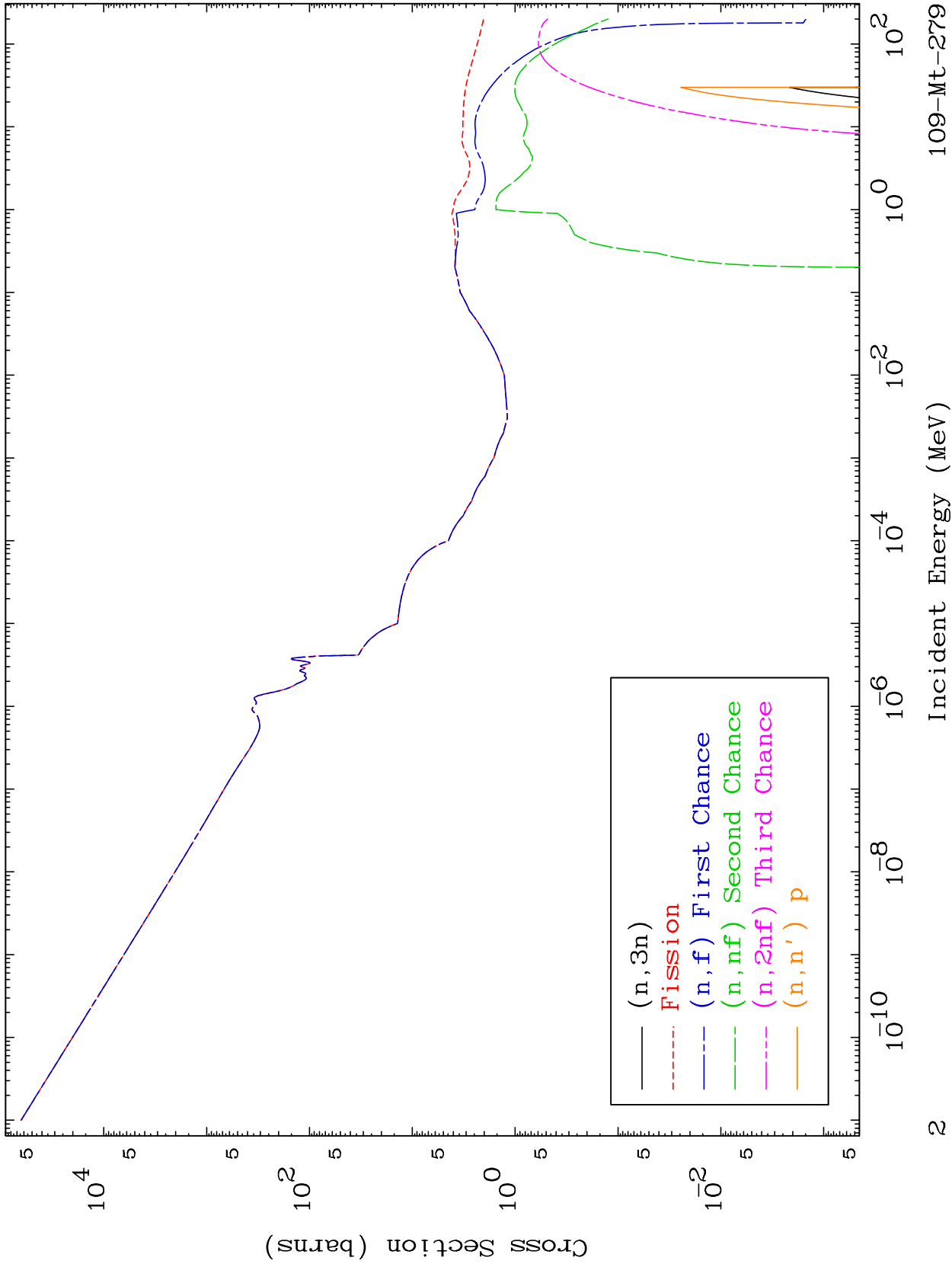
109-Mt-279

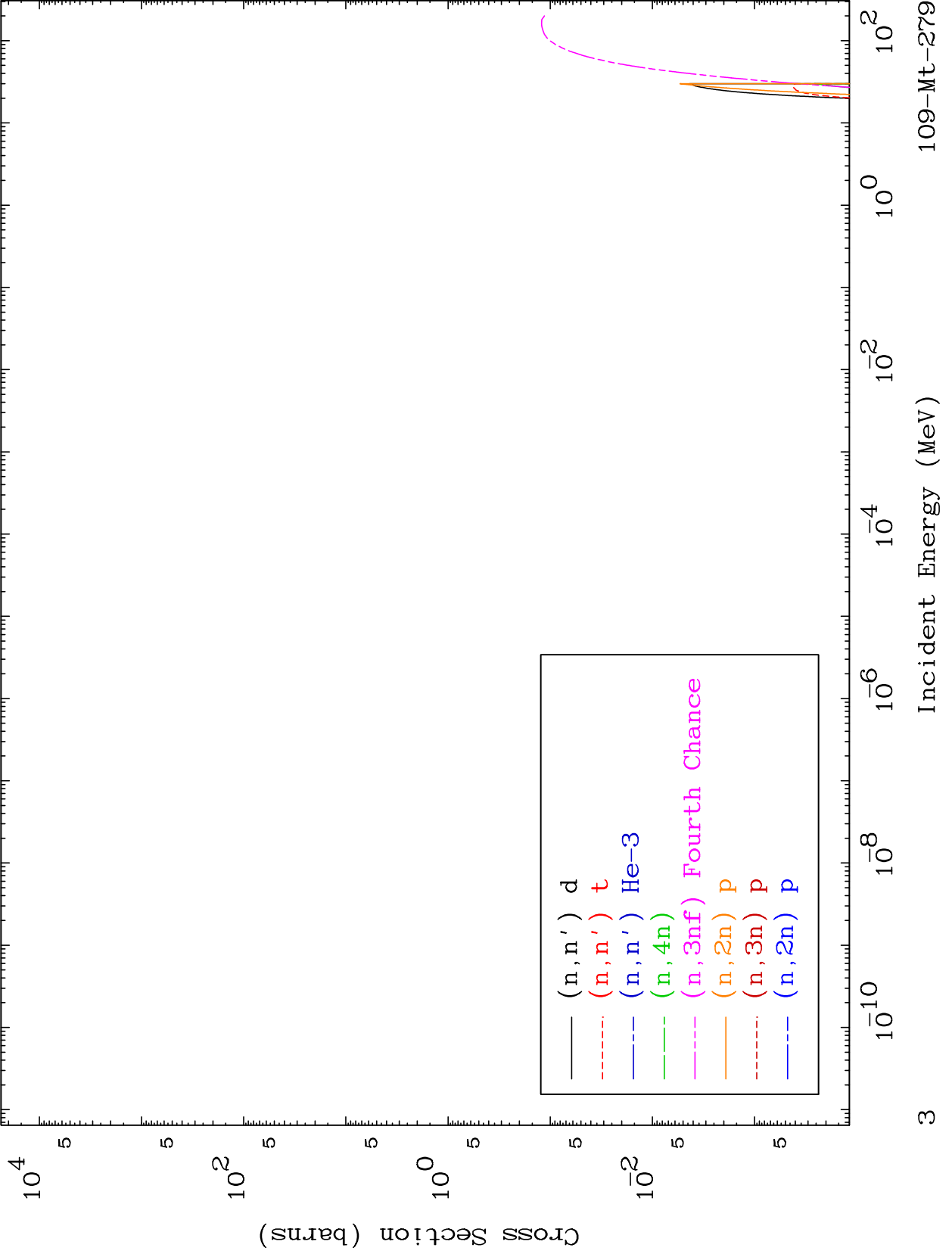


MAT 979

Neutron Absorption  
293 Kelvin Cross Sections

109-Mt-279

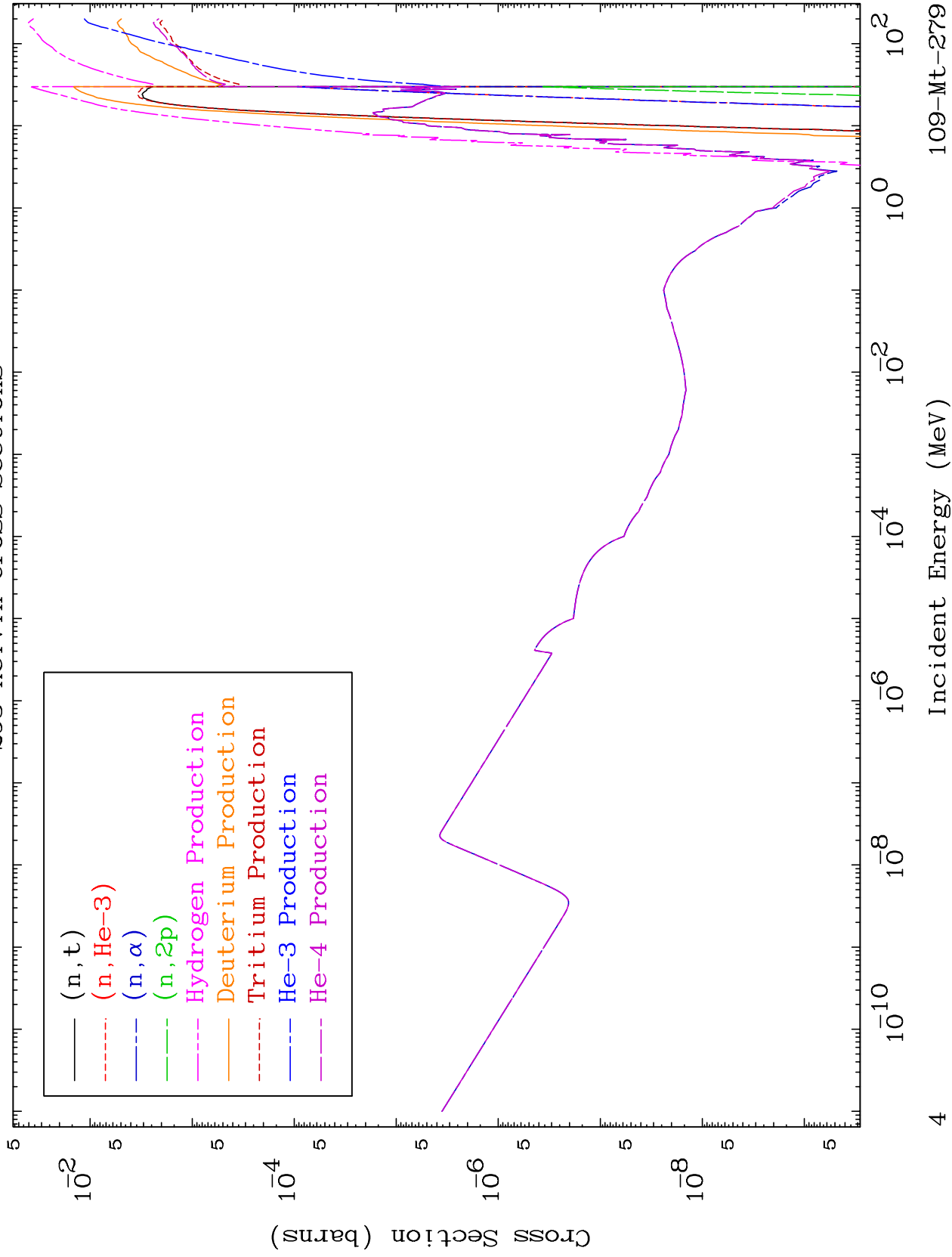




MAT 979

Neutron Absorption  
293 Kelvin Cross Sections

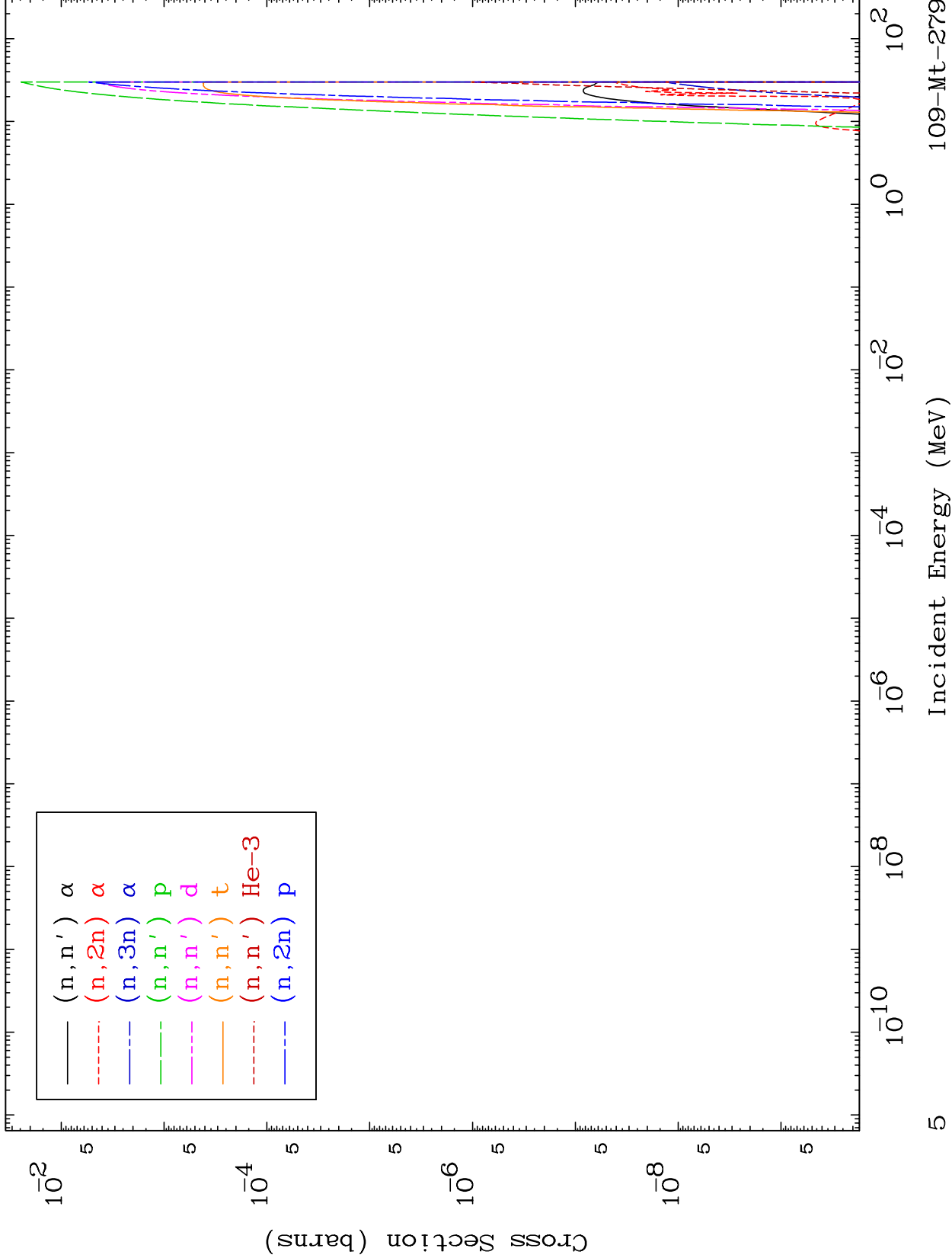
109-Mt-279



MAT 979

Charged Particle  
293 Kelvin Cross Sections

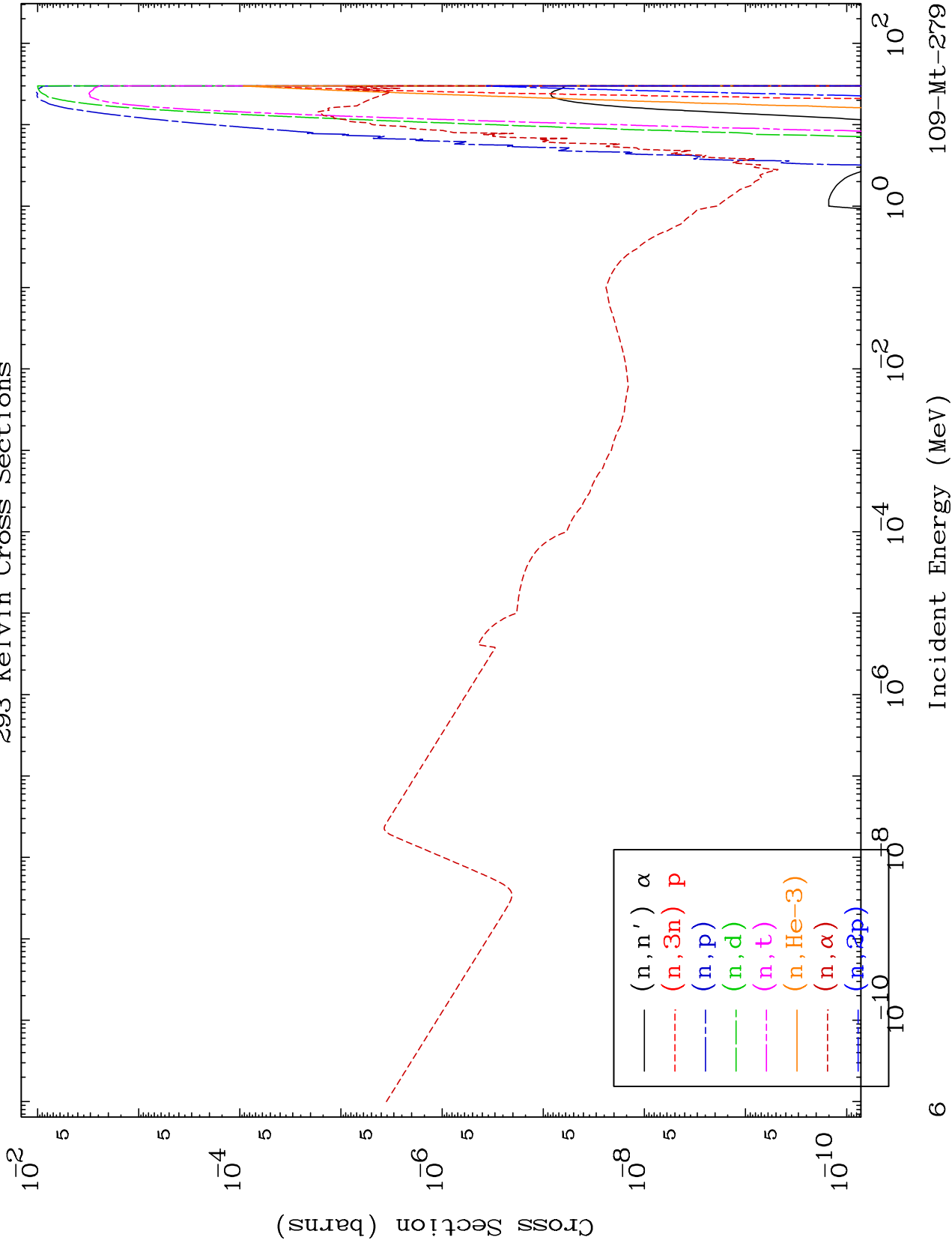
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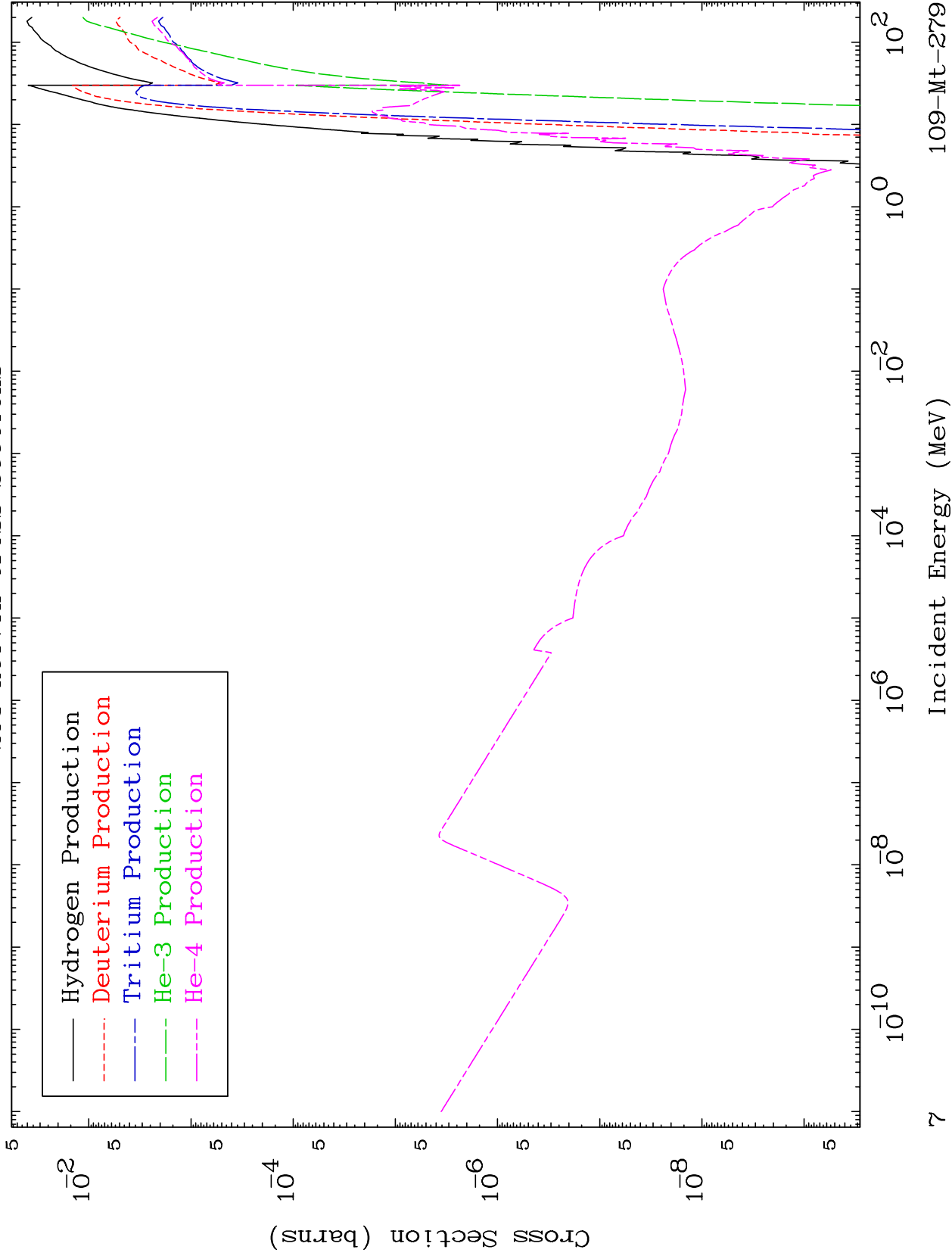


MAT 979

Charged Particle  
293 Kelvin Cross Sections

109-Mt-279

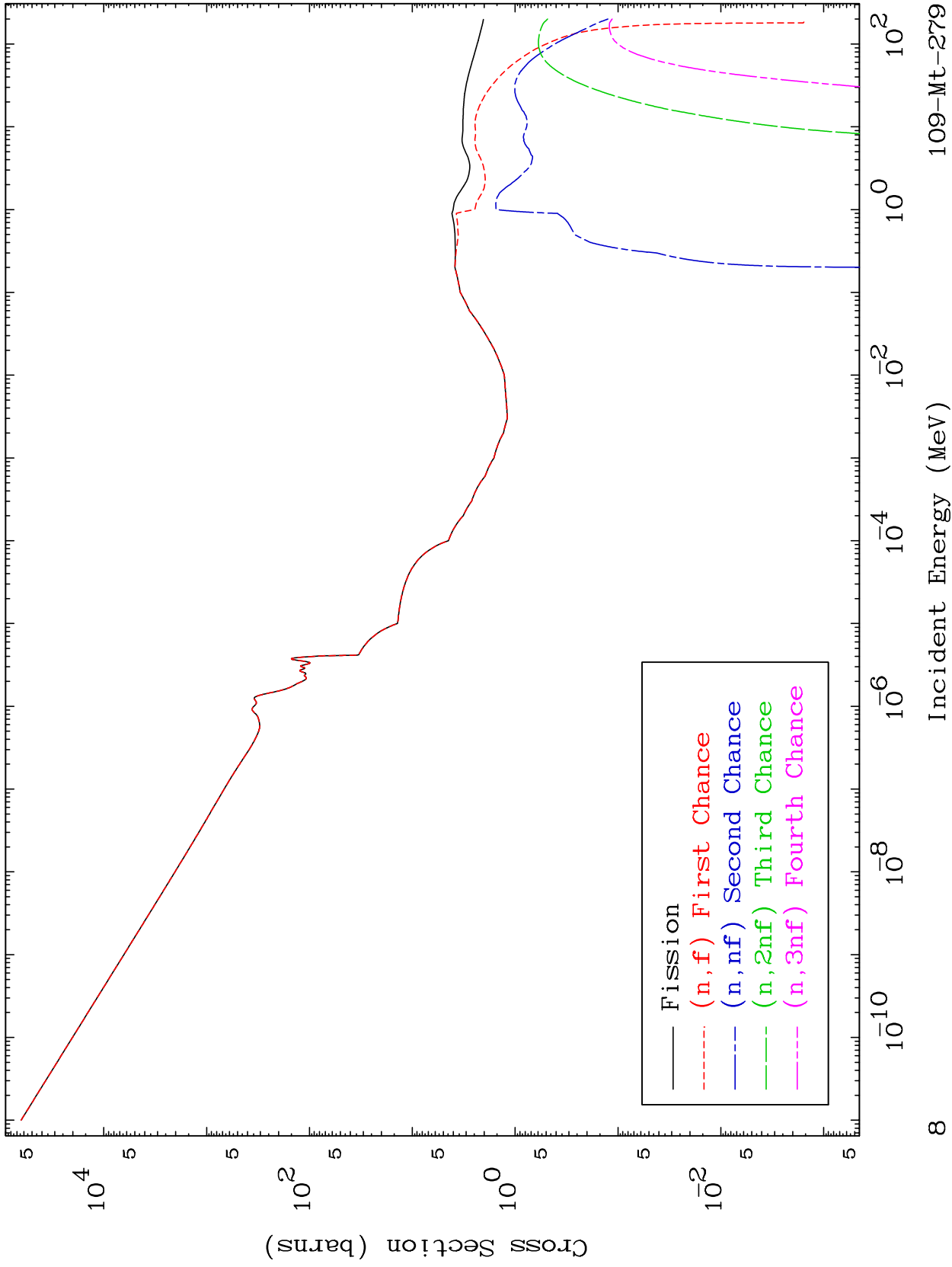




MAT 979

Fission  
293 Kelvin Cross Sections

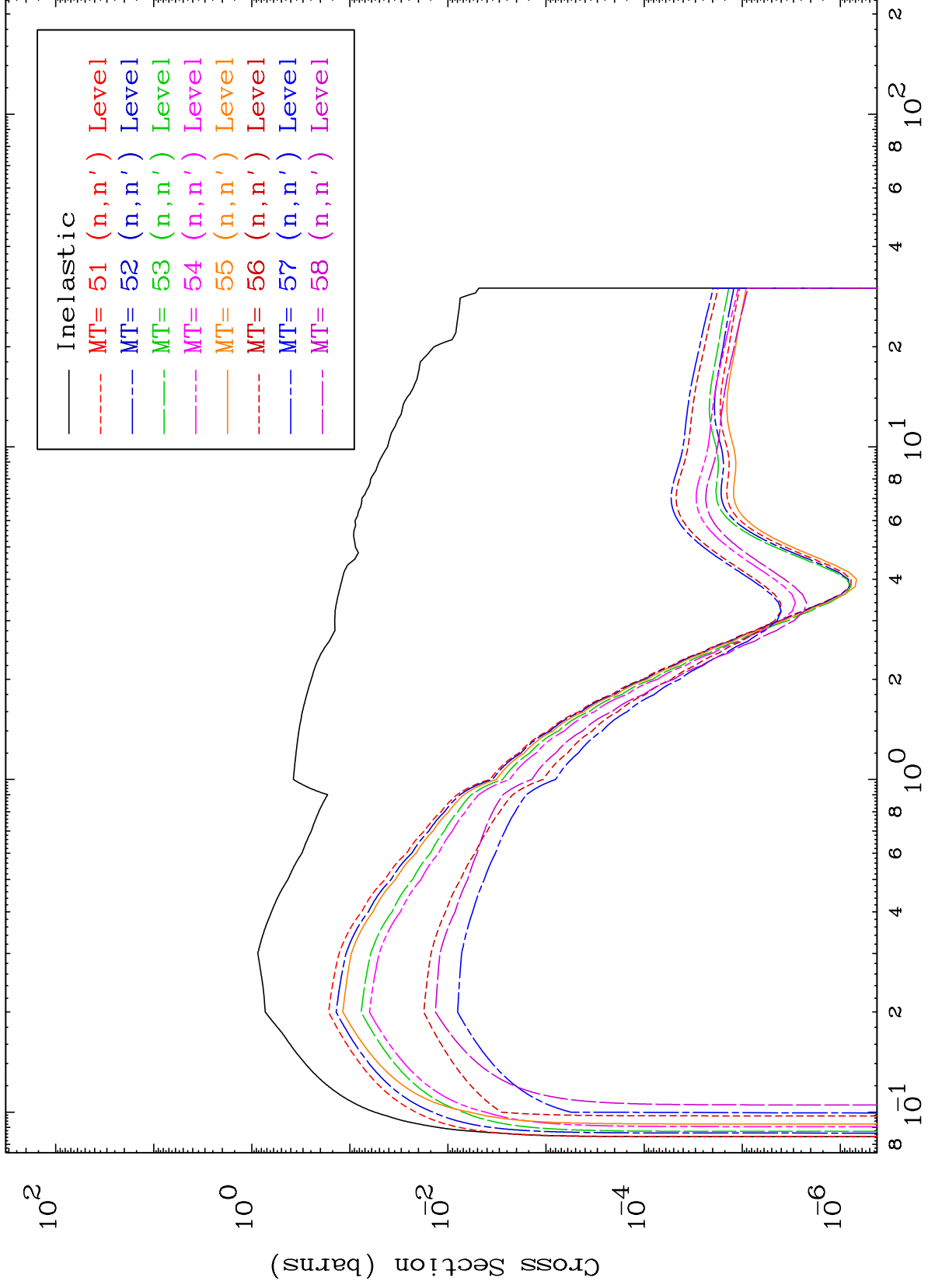
109-Mt-279



MAT 979

(n,n') Levels  
293 Kelvin Cross Sections

109-Mt-279



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Incident Energy (MeV)

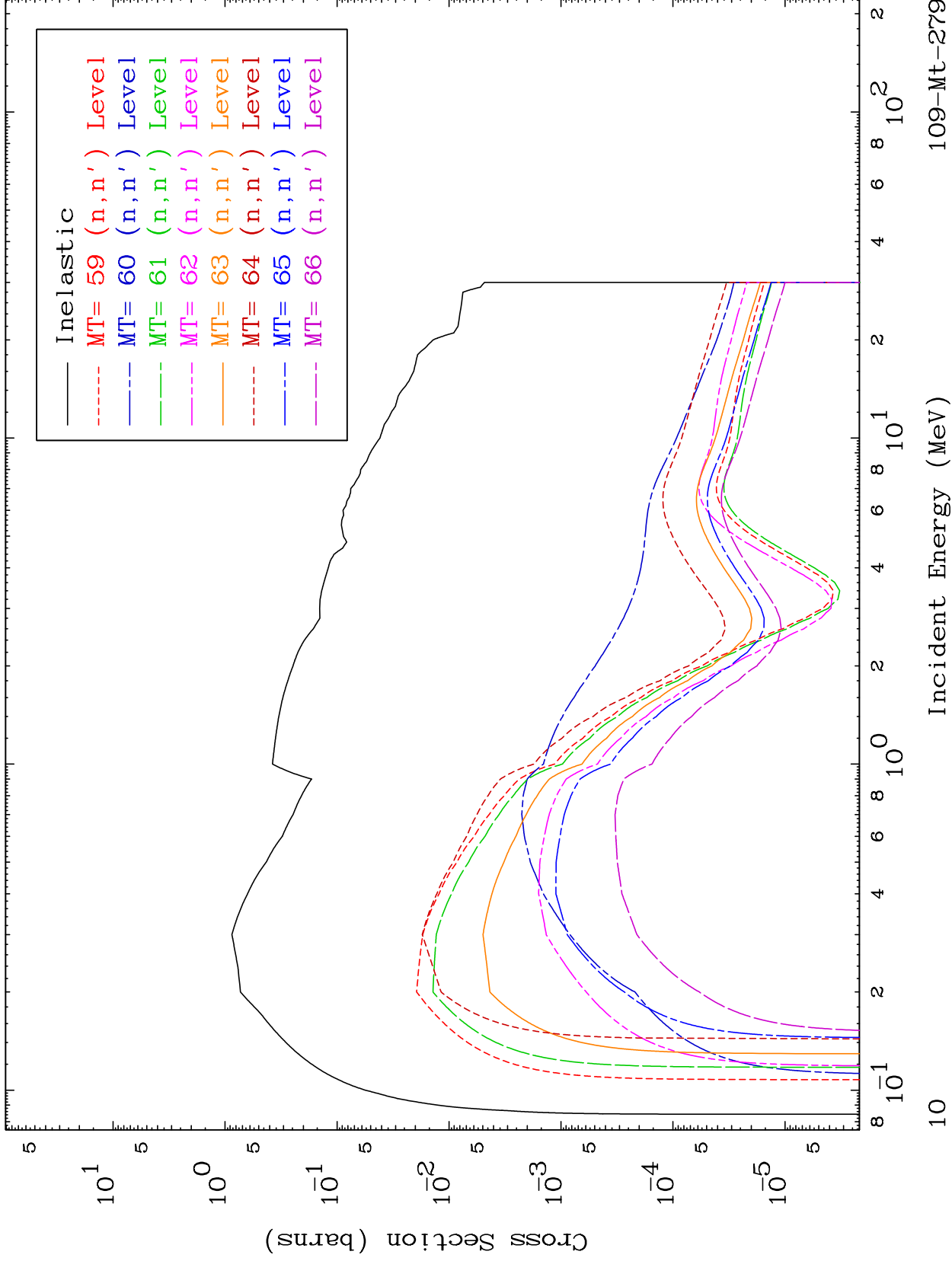
109-Mt-279

MAT 979

(n,n') Levels

109-Mt-279

293 Kelvin Cross Sections



109-Mt-279

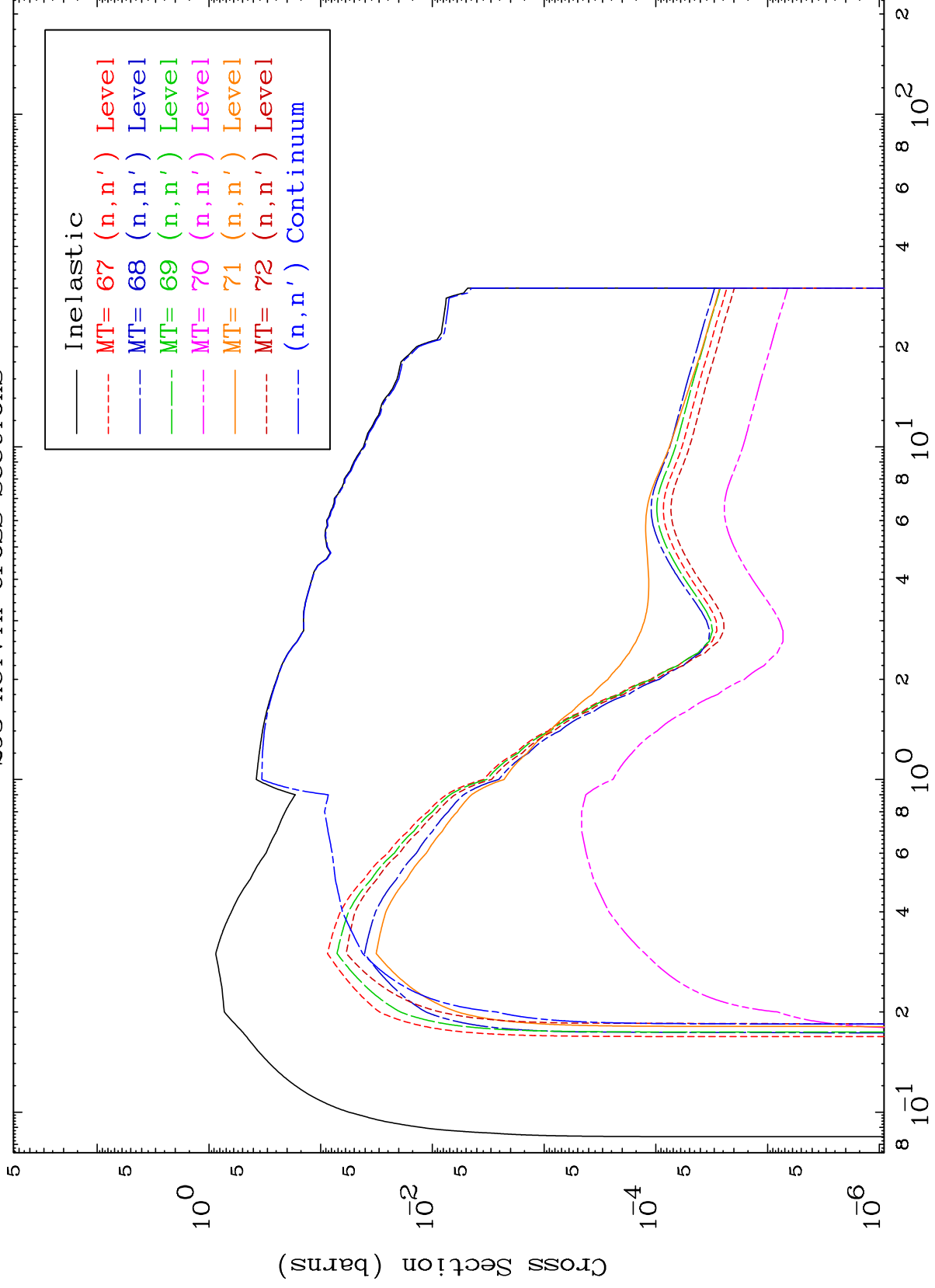
Incident Energy (MeV)

10

MAT 979

(n,n') Levels  
293 Kelvin Cross Sections

109-Mt-279



11

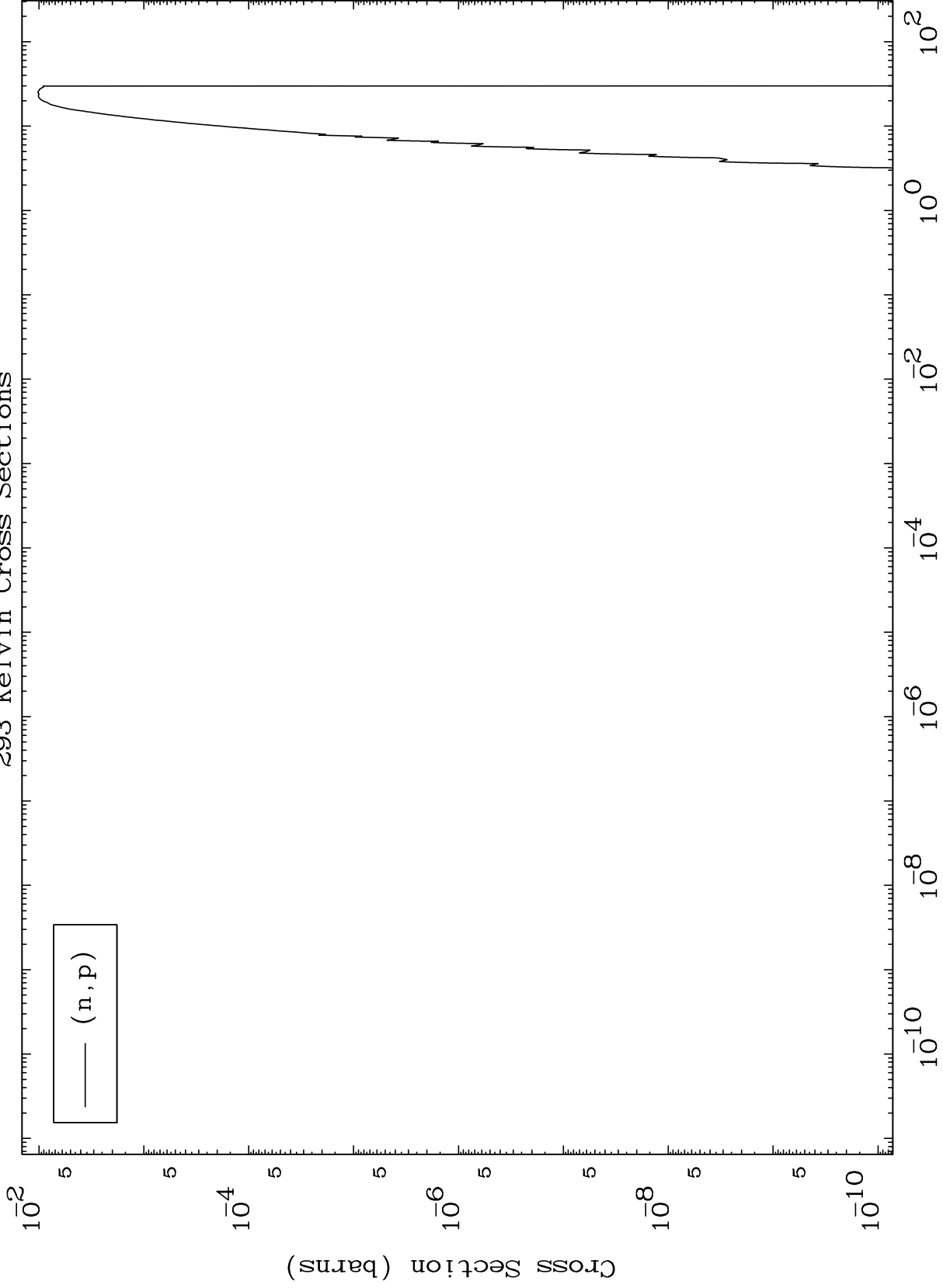
Incident Energy (MeV)

109-Mt-279

MAT 979

(n,p) Levels  
293 Kelvin Cross Sections

109-Mt-279



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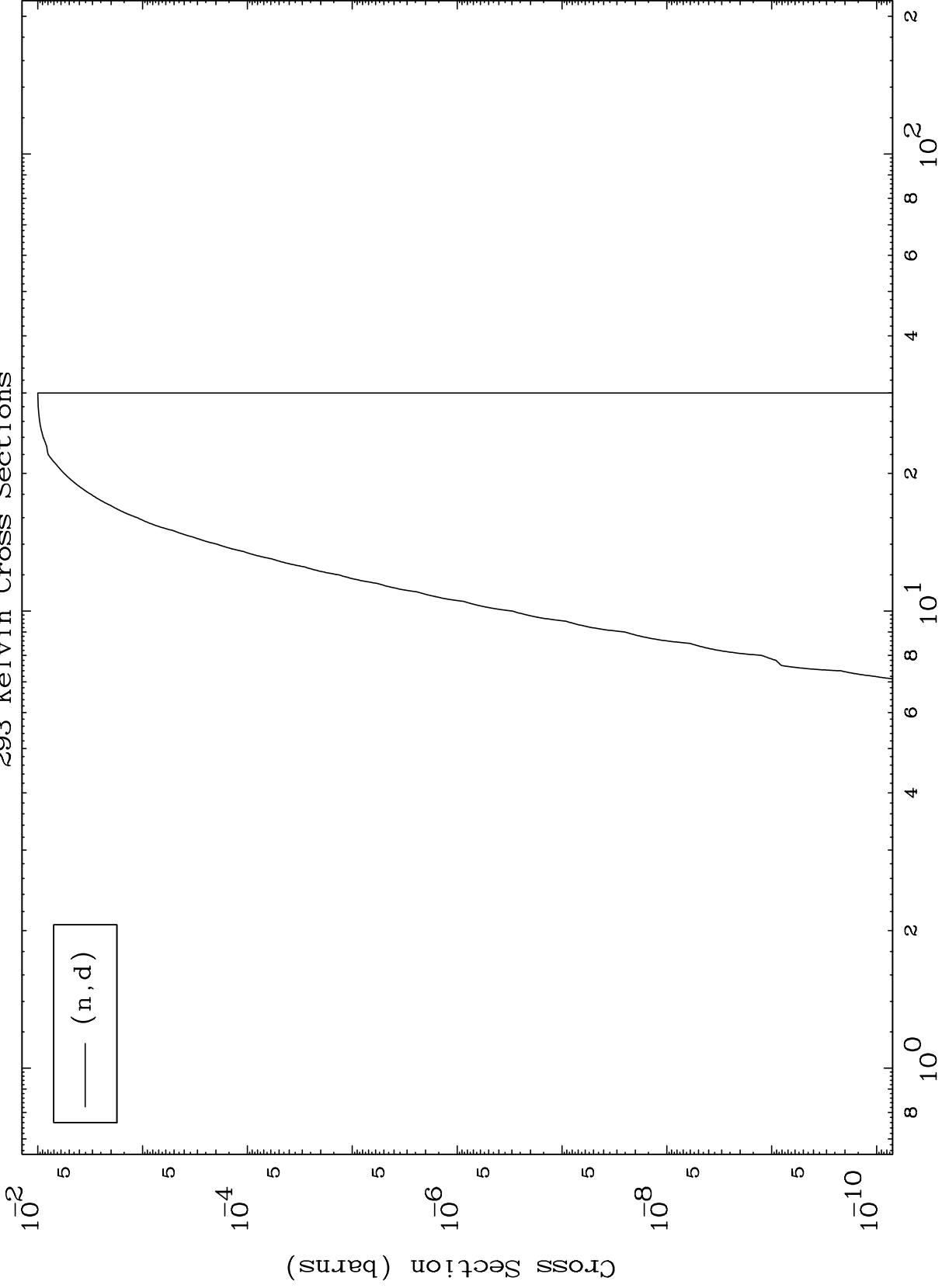
Incident Energy (MeV)

109-Mt-279

MAT 979

(n,d) Levels  
293 Kelvin Cross Sections

109-Mt-279



13

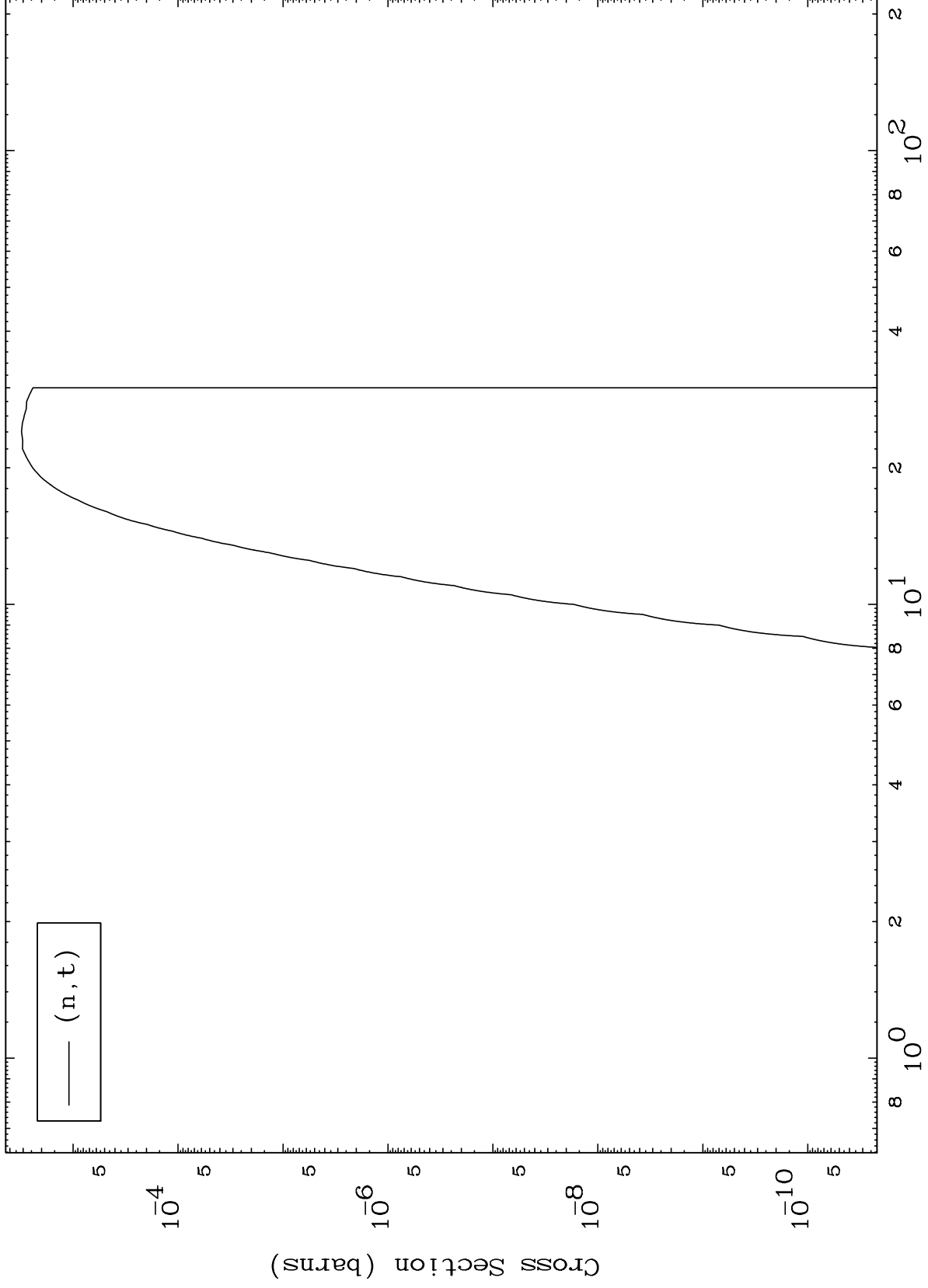
Incident Energy (MeV)

109-Mt-279

MAT 979

(n,t) Levels  
293 Kelvin Cross Sections

109-Mt-279



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Incident Energy (MeV)

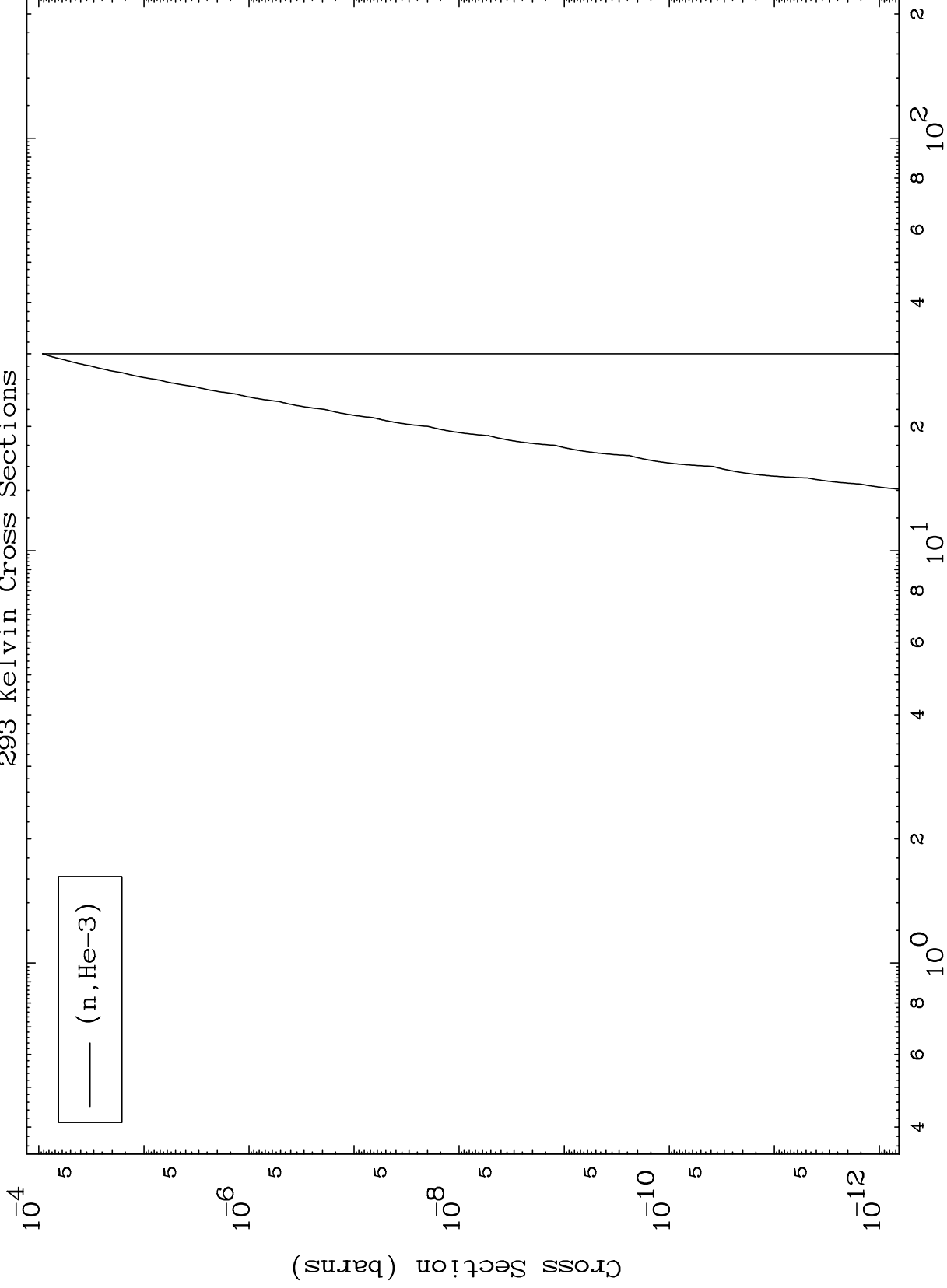
109-Mt-279

MAT 979

(n,He3) Levels

109-Mt-279

293 Kelvin Cross Sections



15

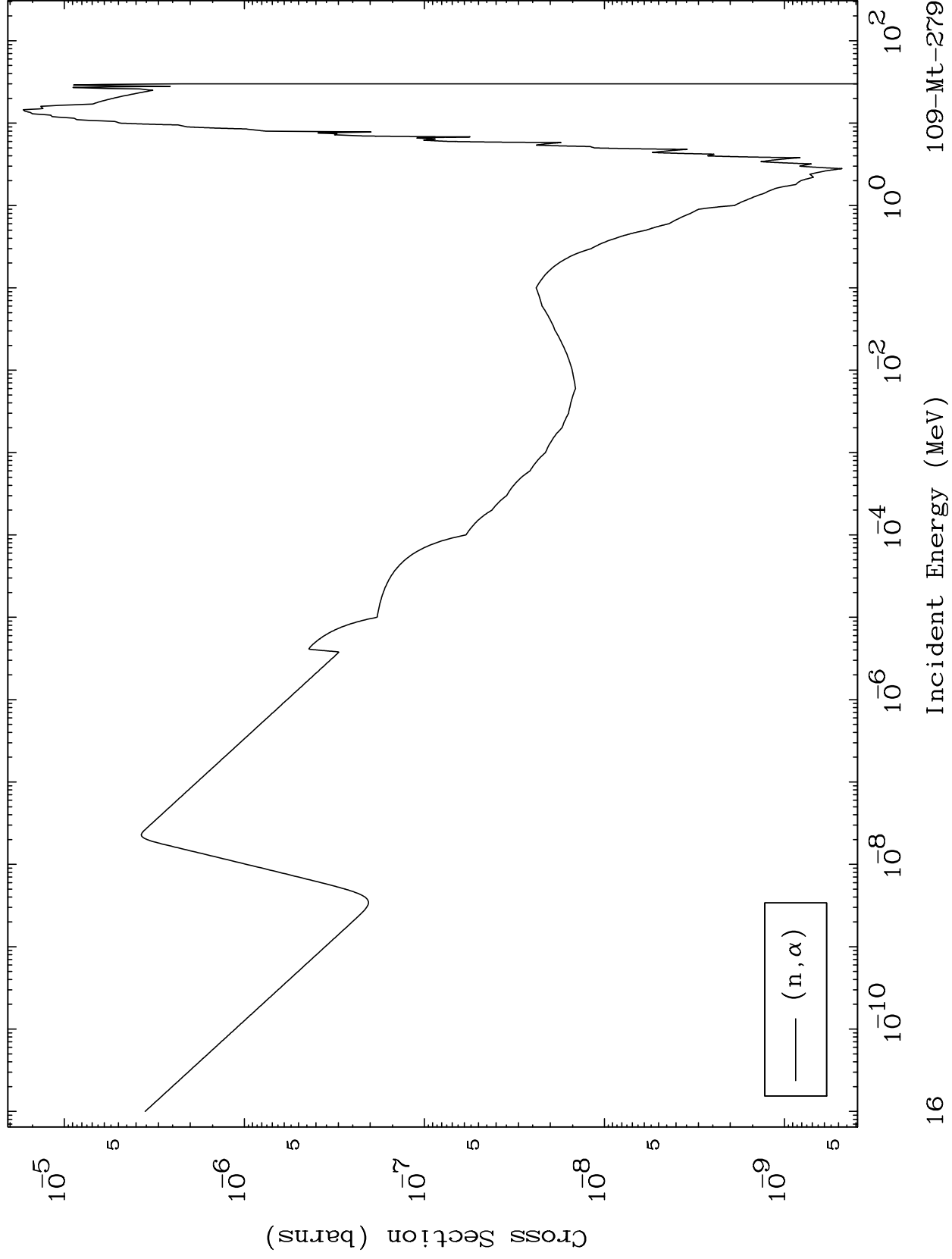
Incident Energy (MeV)

109-Mt-279

MAT 979

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

109-Mt-279

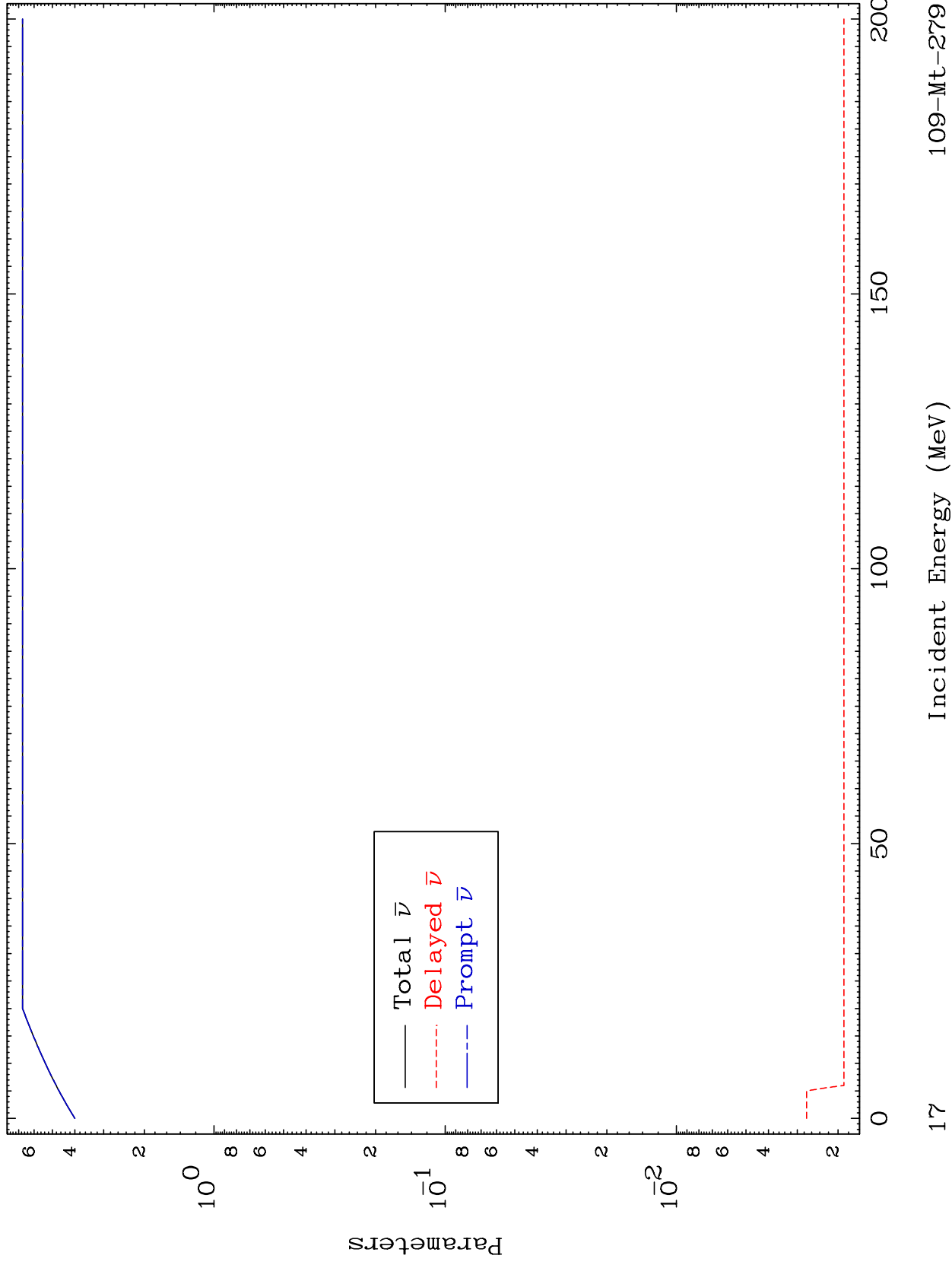


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Energy Release  
Parameters

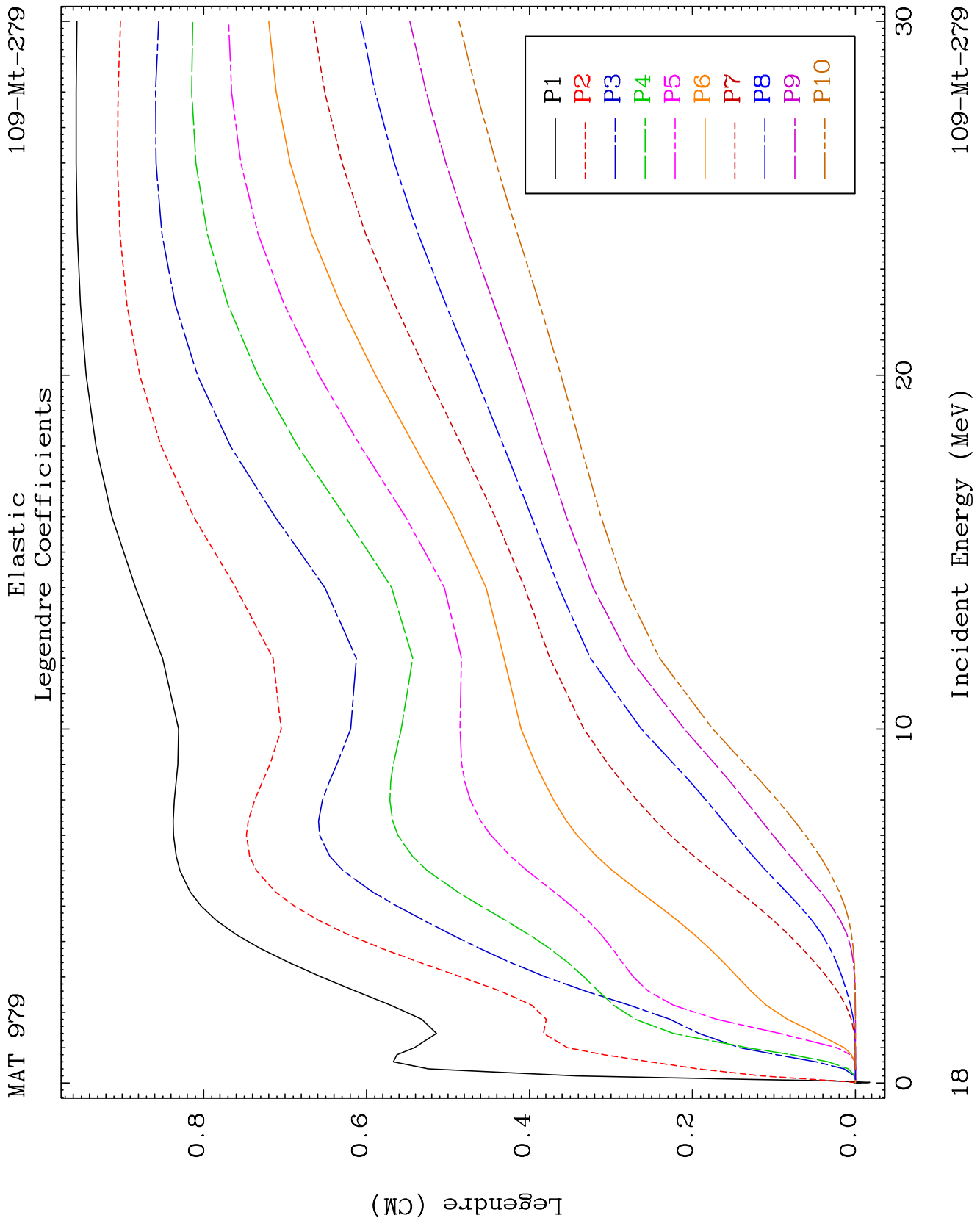
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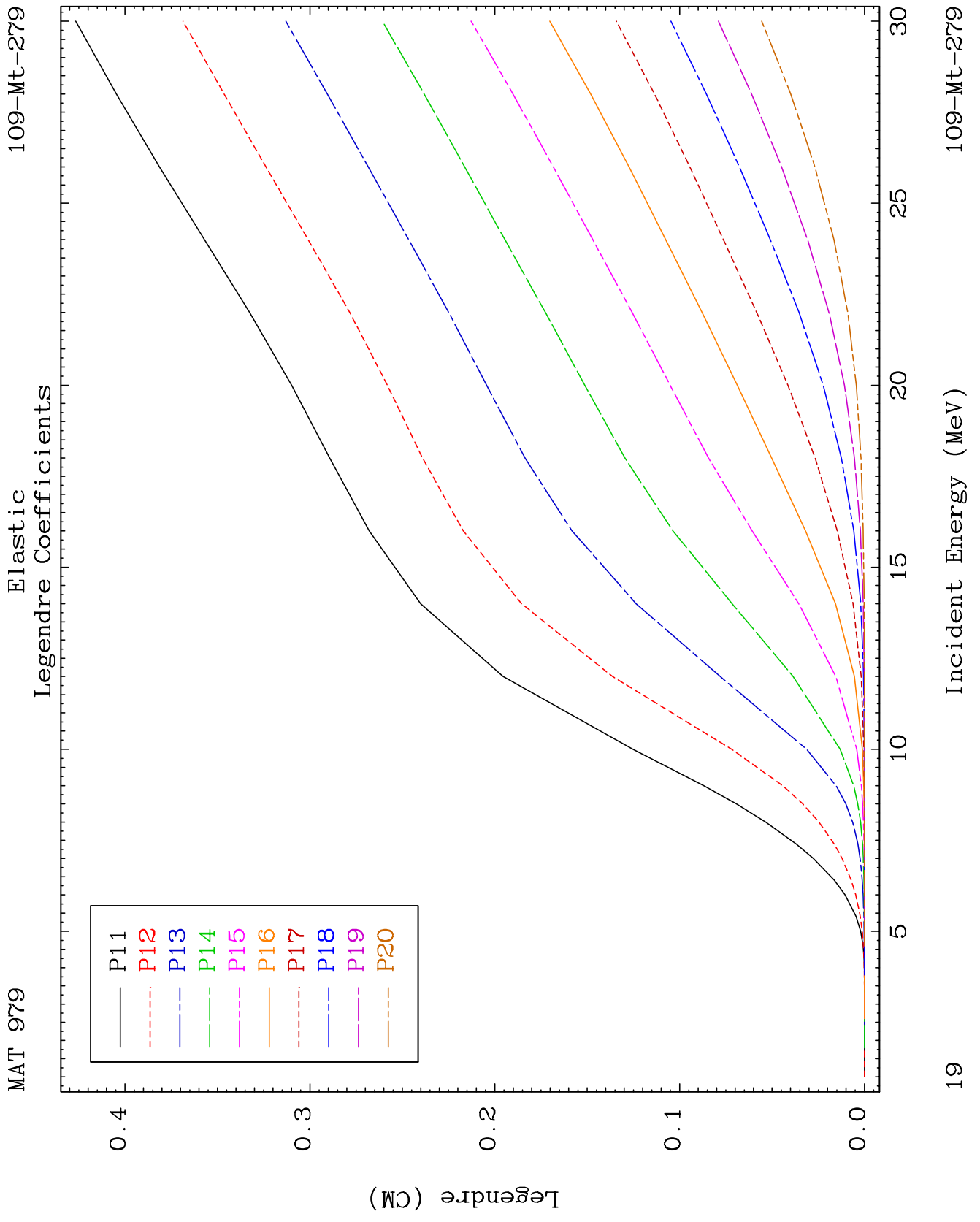


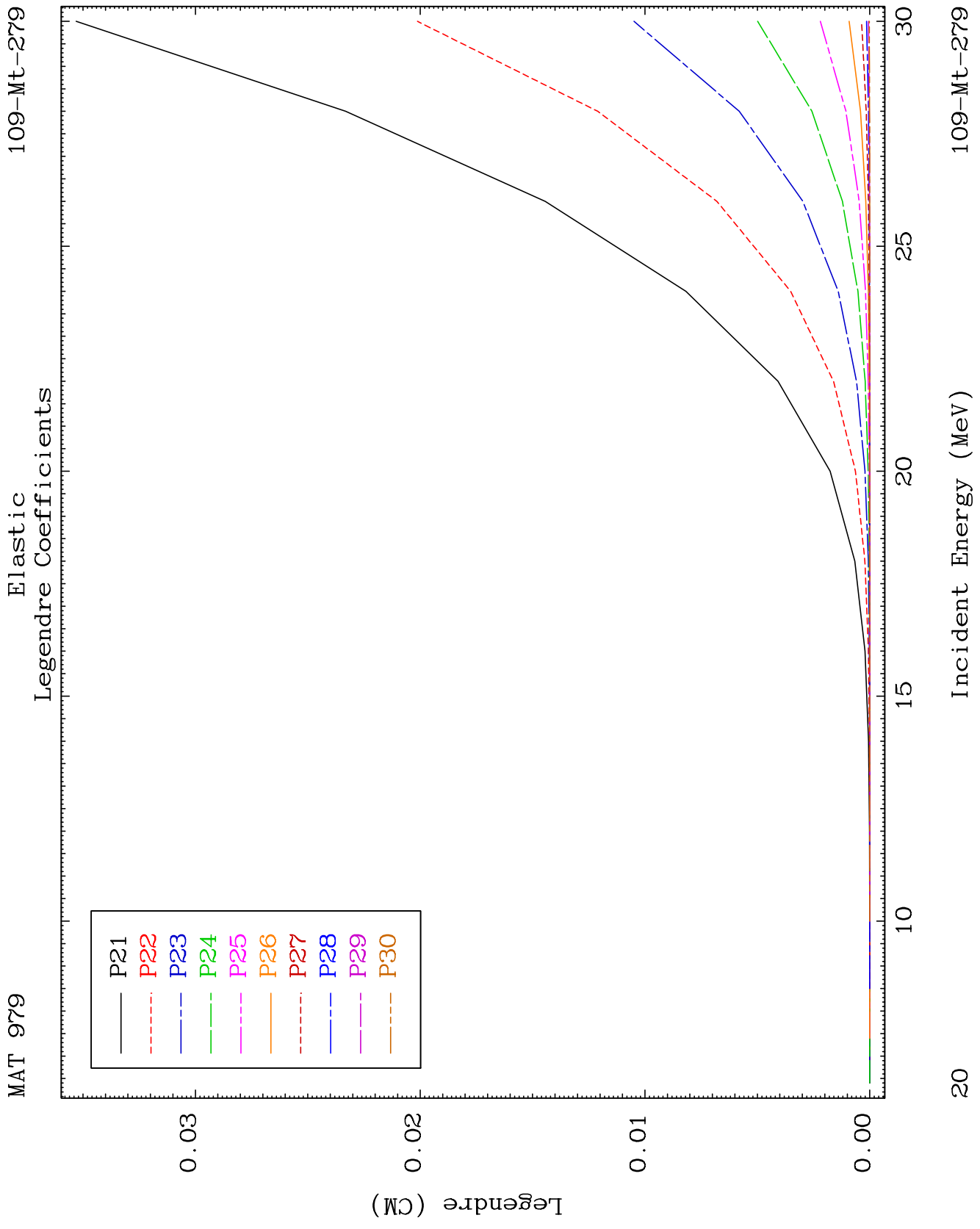
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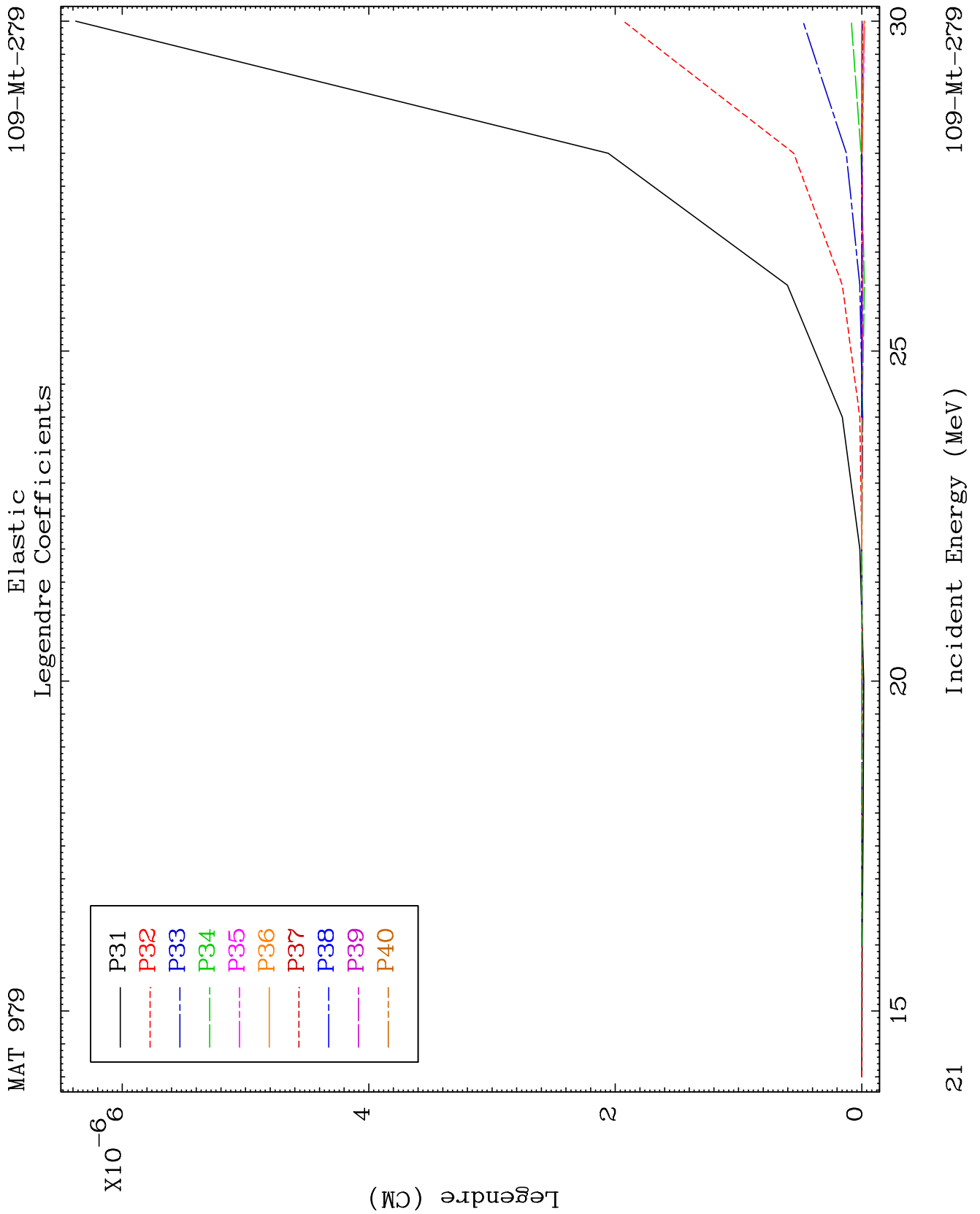
Incident Energy (MeV)

109-Mt-279





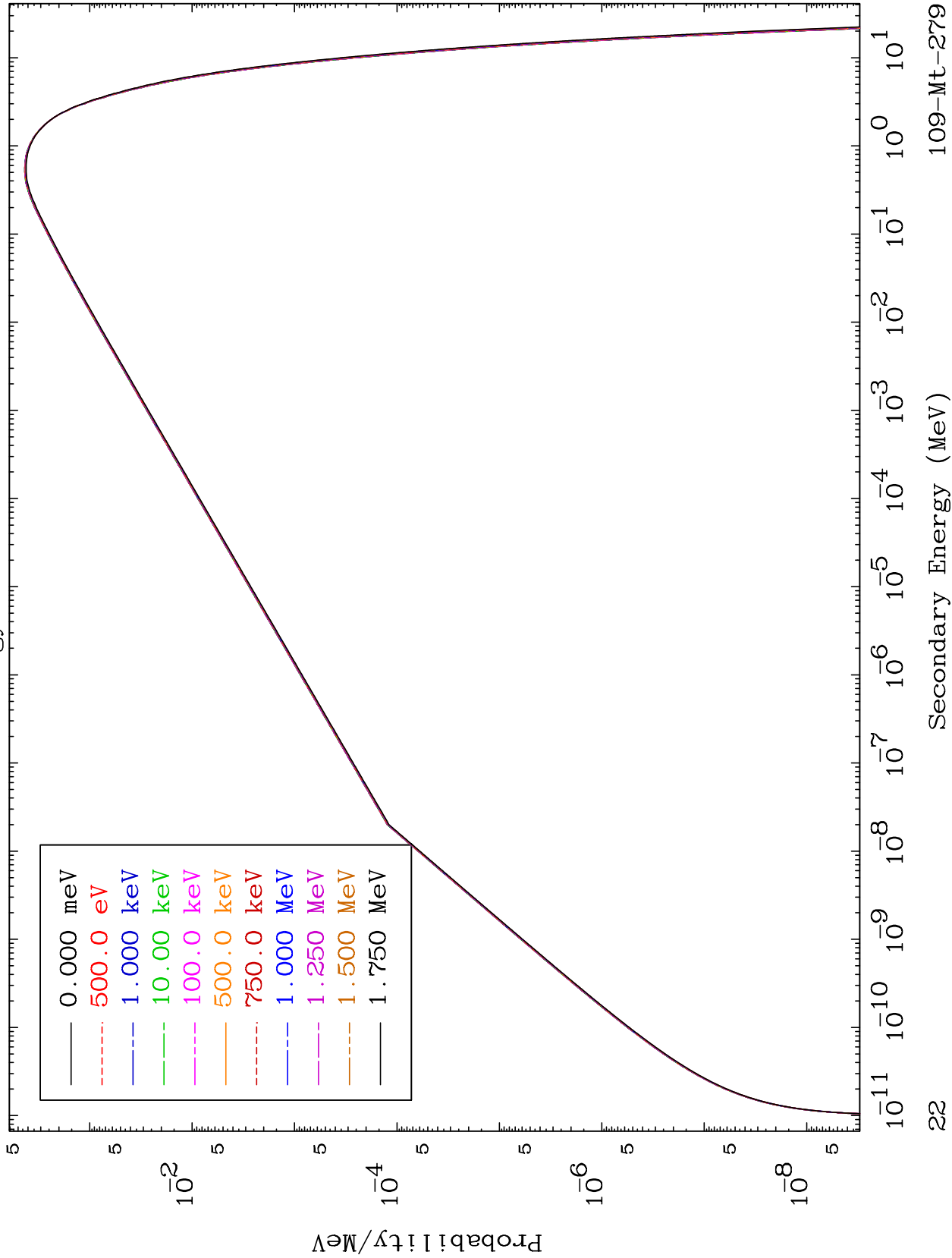




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### Fission Energy Distributions

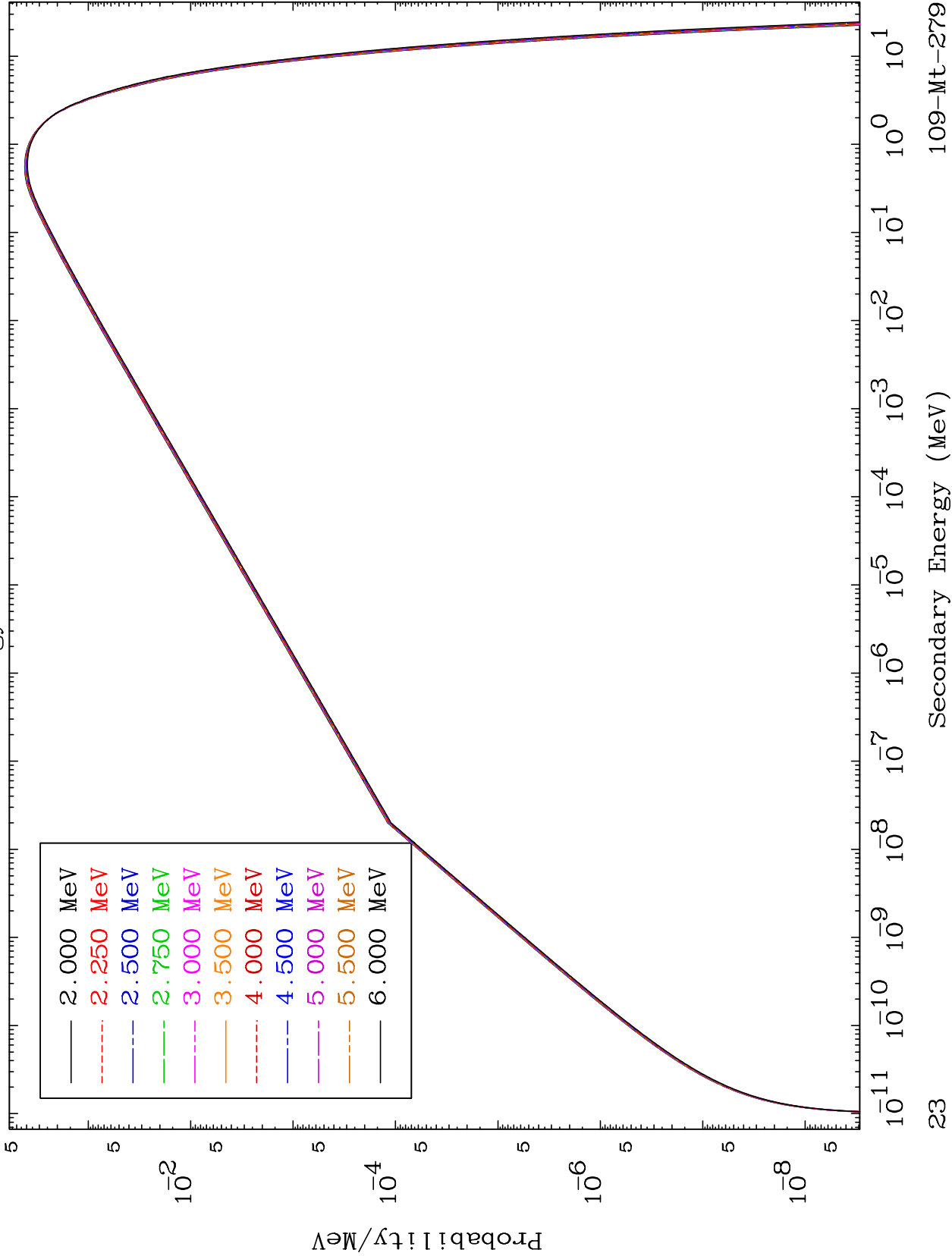
109-Mt-279



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### Fission Energy Distributions

109-Mt-279



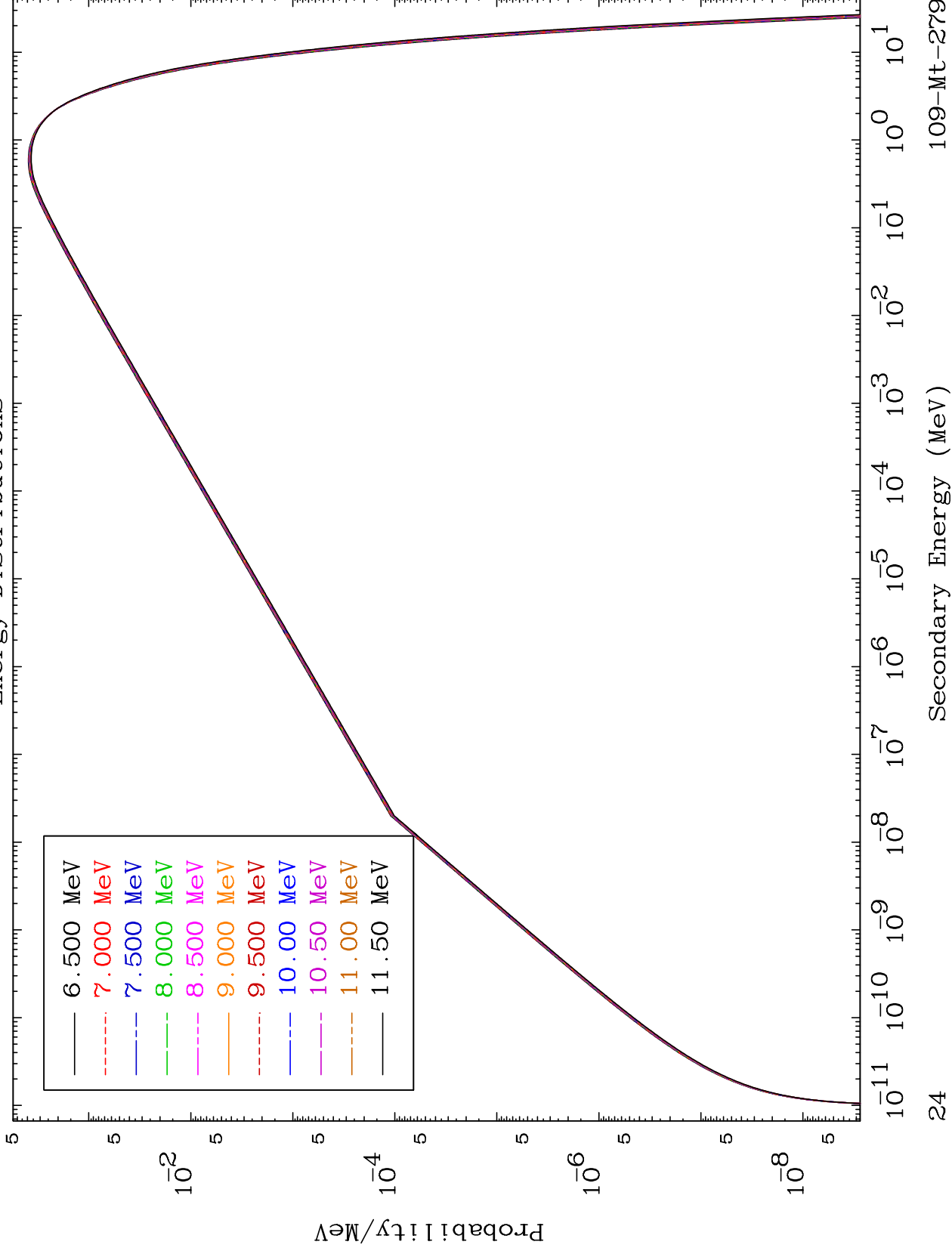
109-Mt-279

Secondary Energy (MeV)

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### Fission Energy Distributions

109-Mt-279



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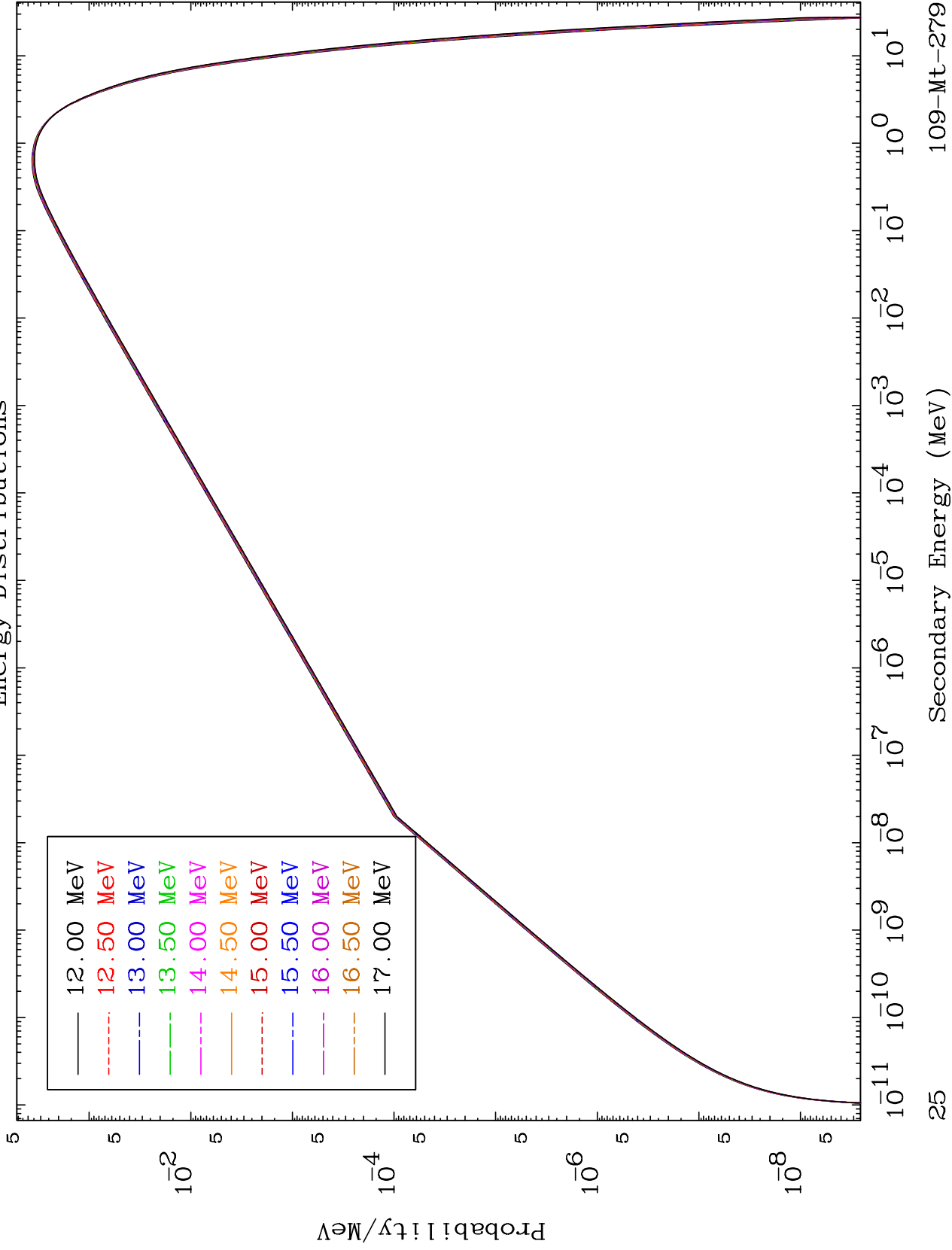
Secondary Energy (MeV)

109-Mt-279

MAT 979

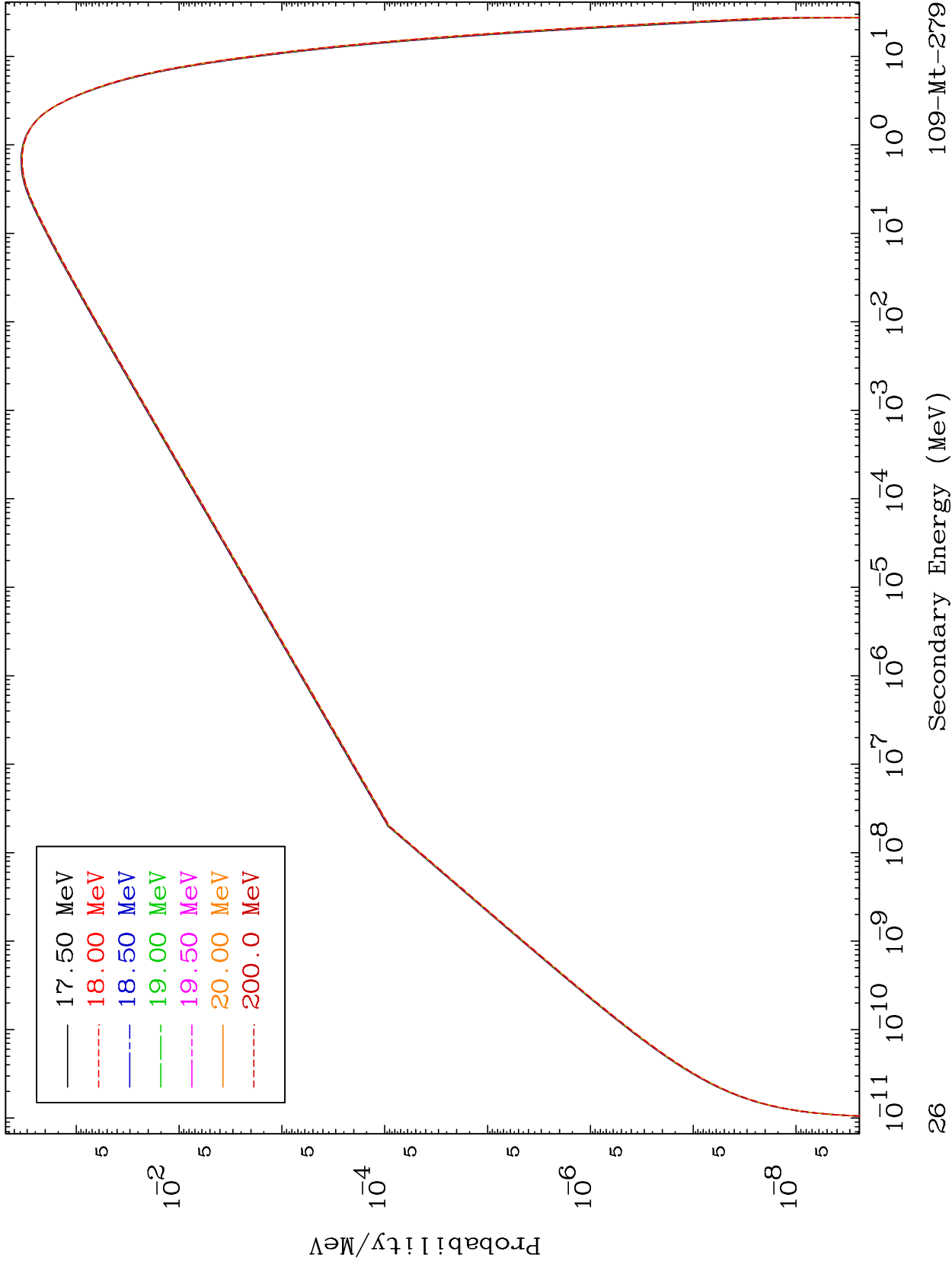
### Fission Energy Distributions

109-Mt-279



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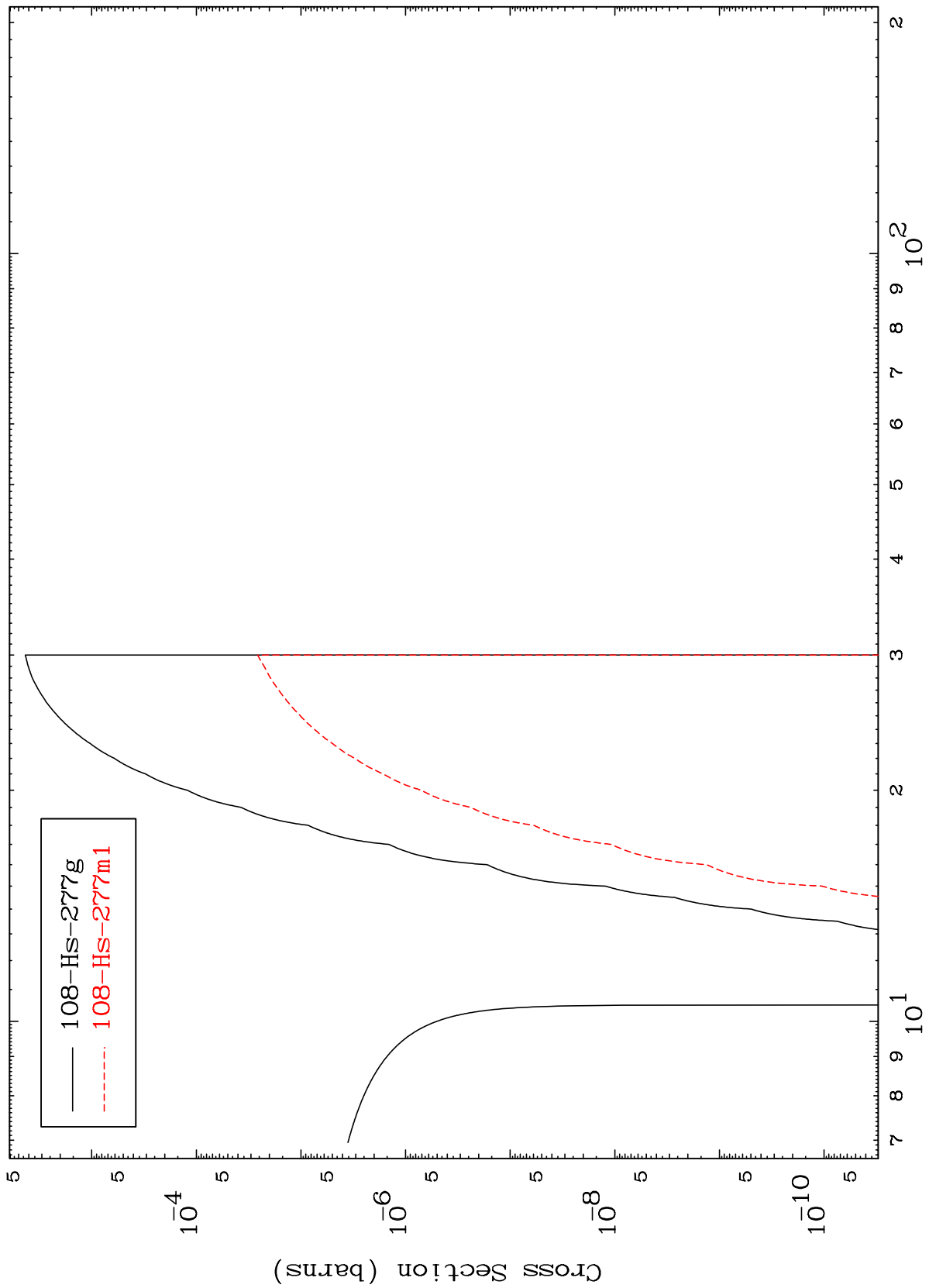
Fission Energy Distributions



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(n,n') d  
Radionuclide Production Cross Section



27

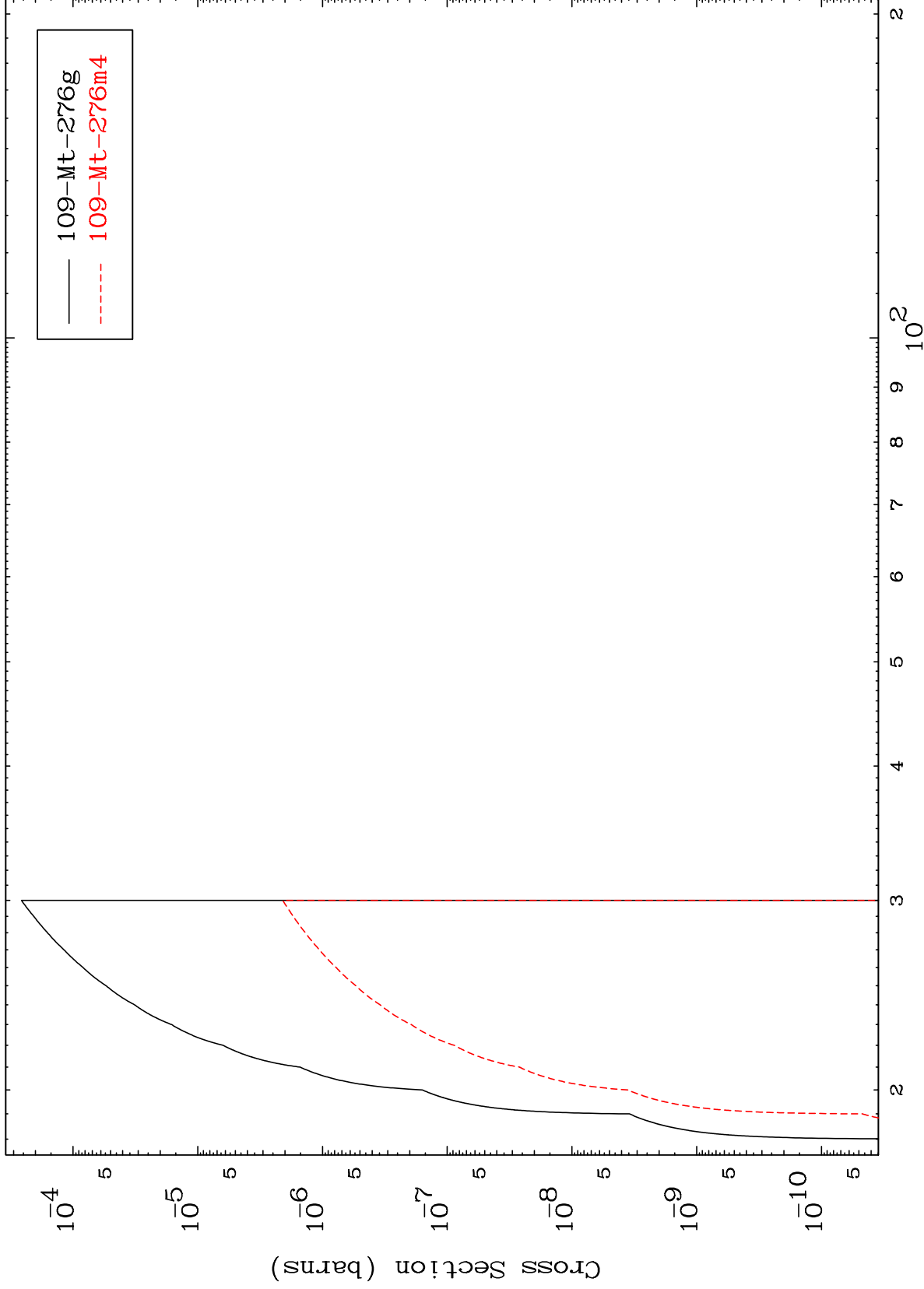
Incident Energy (MeV)

109-Mt-279

MAT 979

109-Mt-279

(n,4n)  
Radionuclide Production Cross Section



28

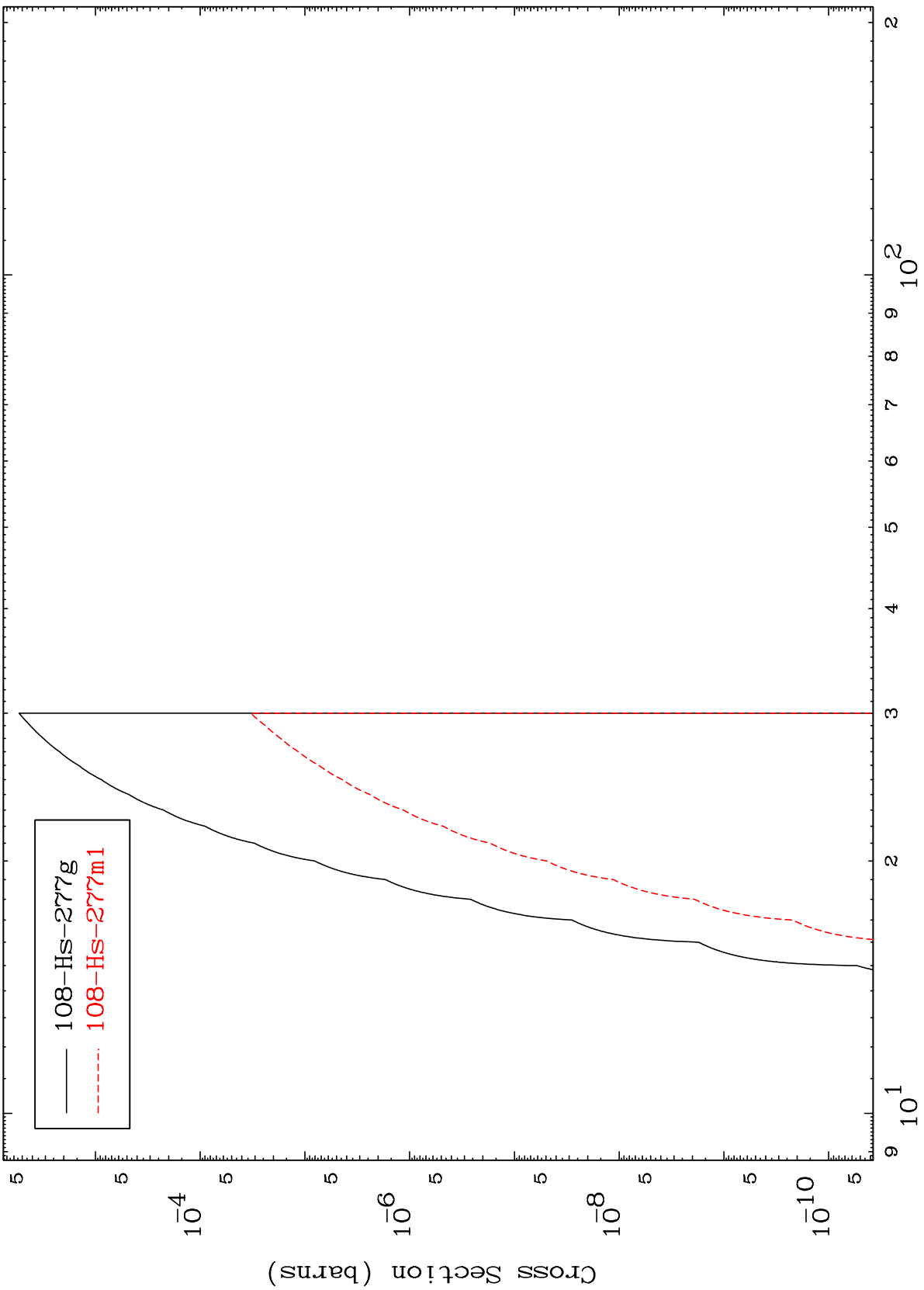
Incident Energy (MeV)

109-Mt-279

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109-Mt-279

(n,2n) p  
Radionuclide Production Cross Section



— 108-Hs-277g  
- - - 108-Hs-277m1

109-Mt-279

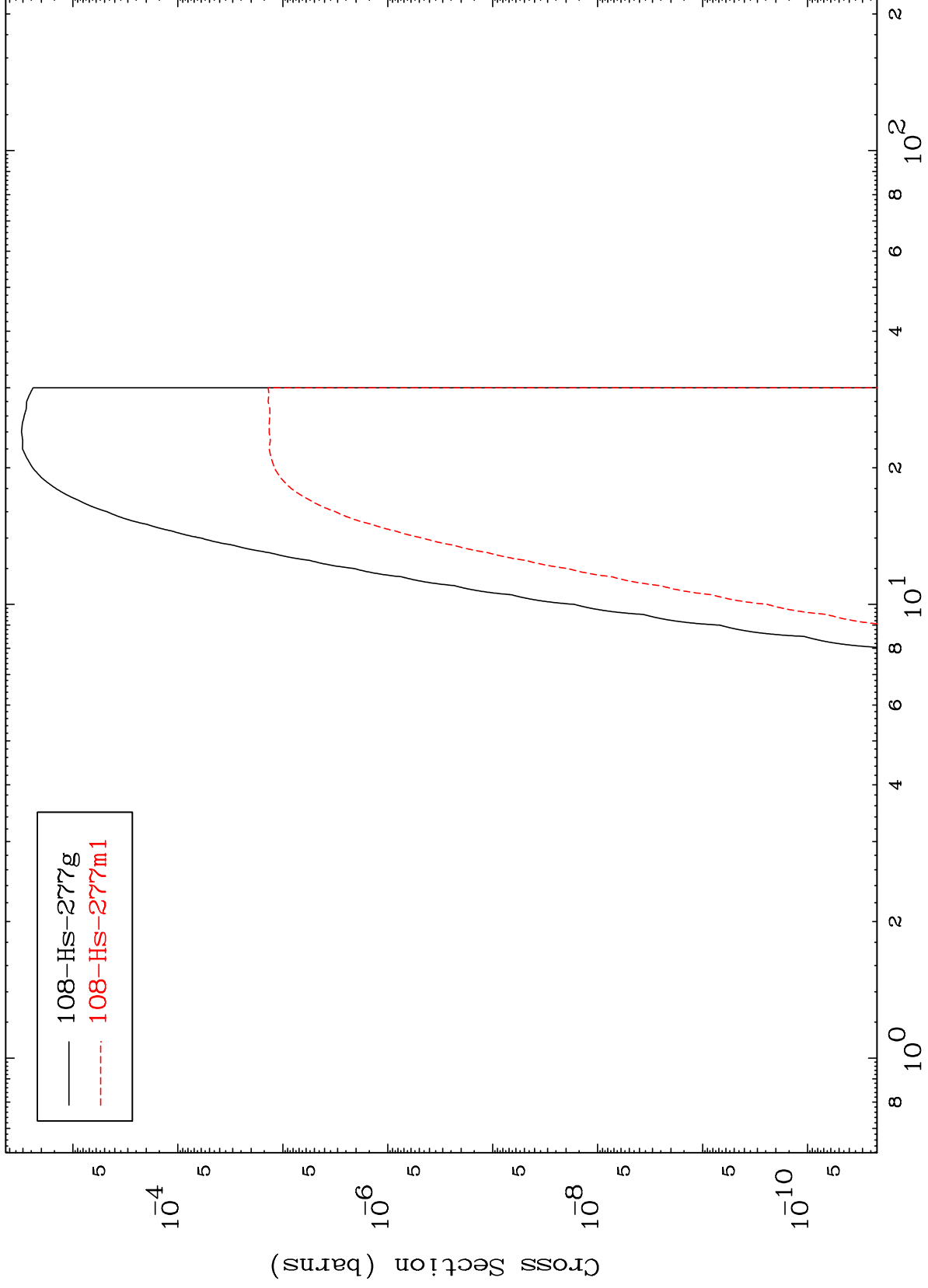
Incident Energy (MeV)

29

MAT 979

109-Mt-279

(n, t)  
Radionuclide Production Cross Section



30

109-Mt-279

Incident Energy (MeV)