

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

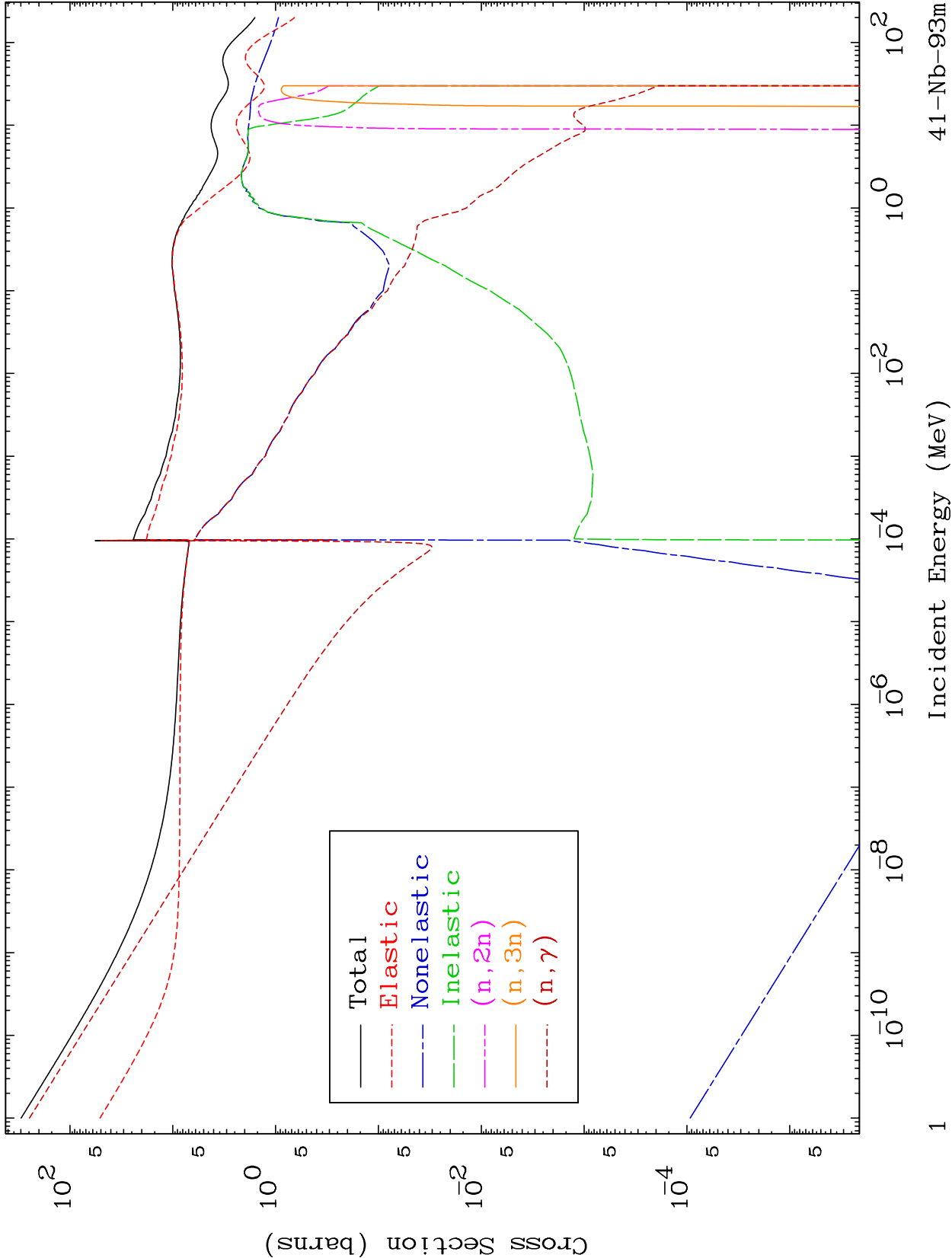
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 4126

Neutron Major
293 Kelvin Cross Sections

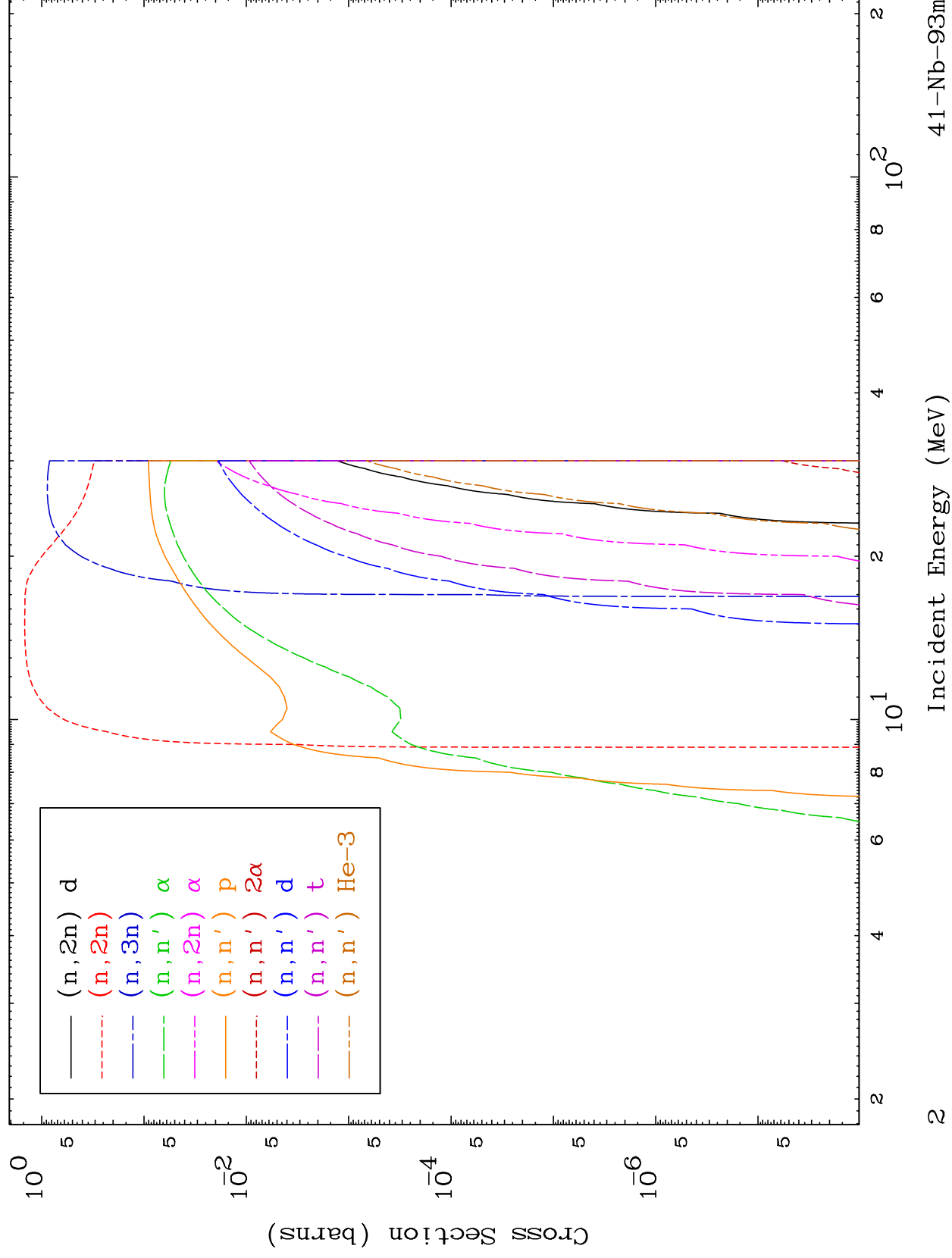
41-Nb-93m

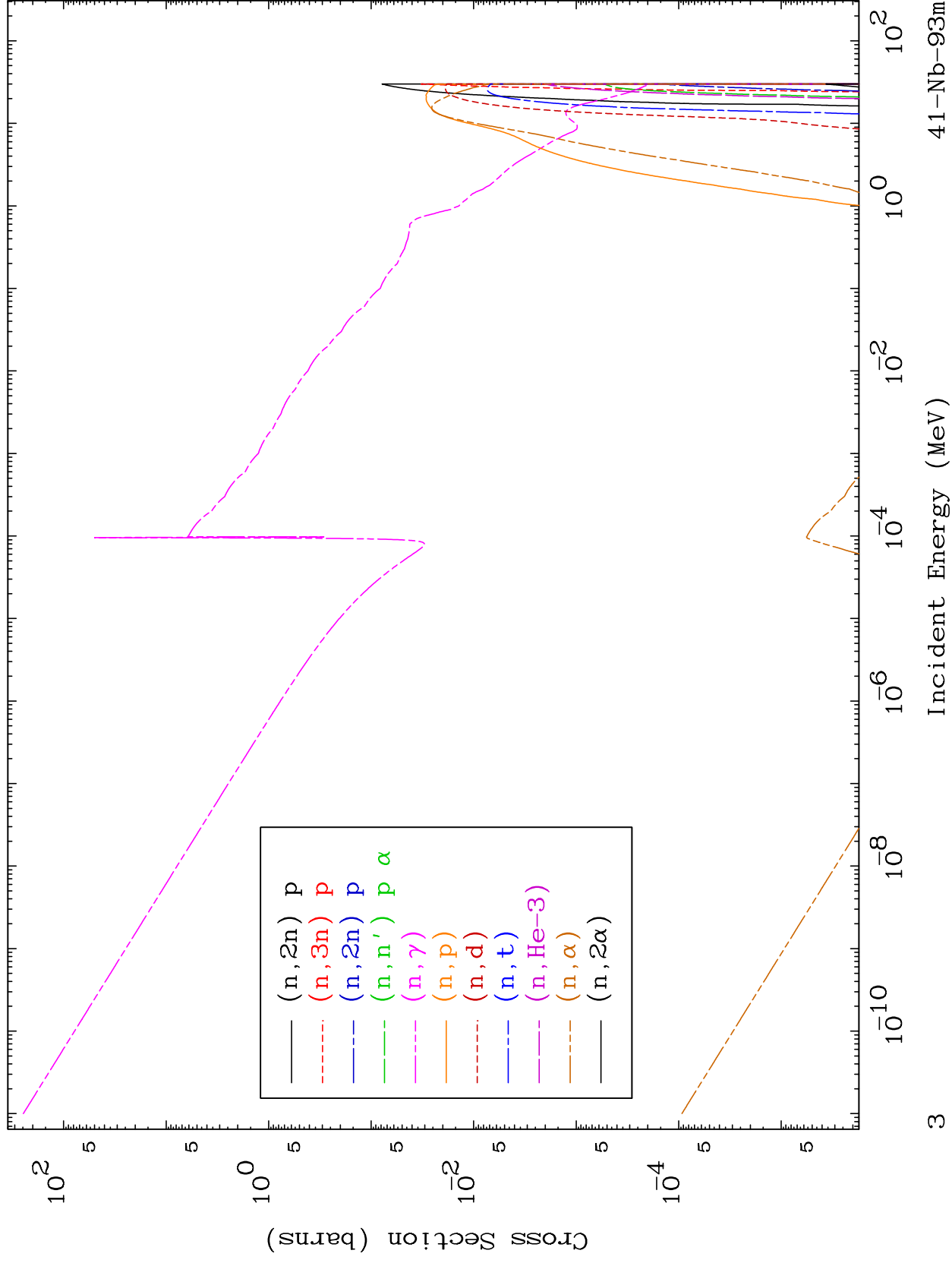


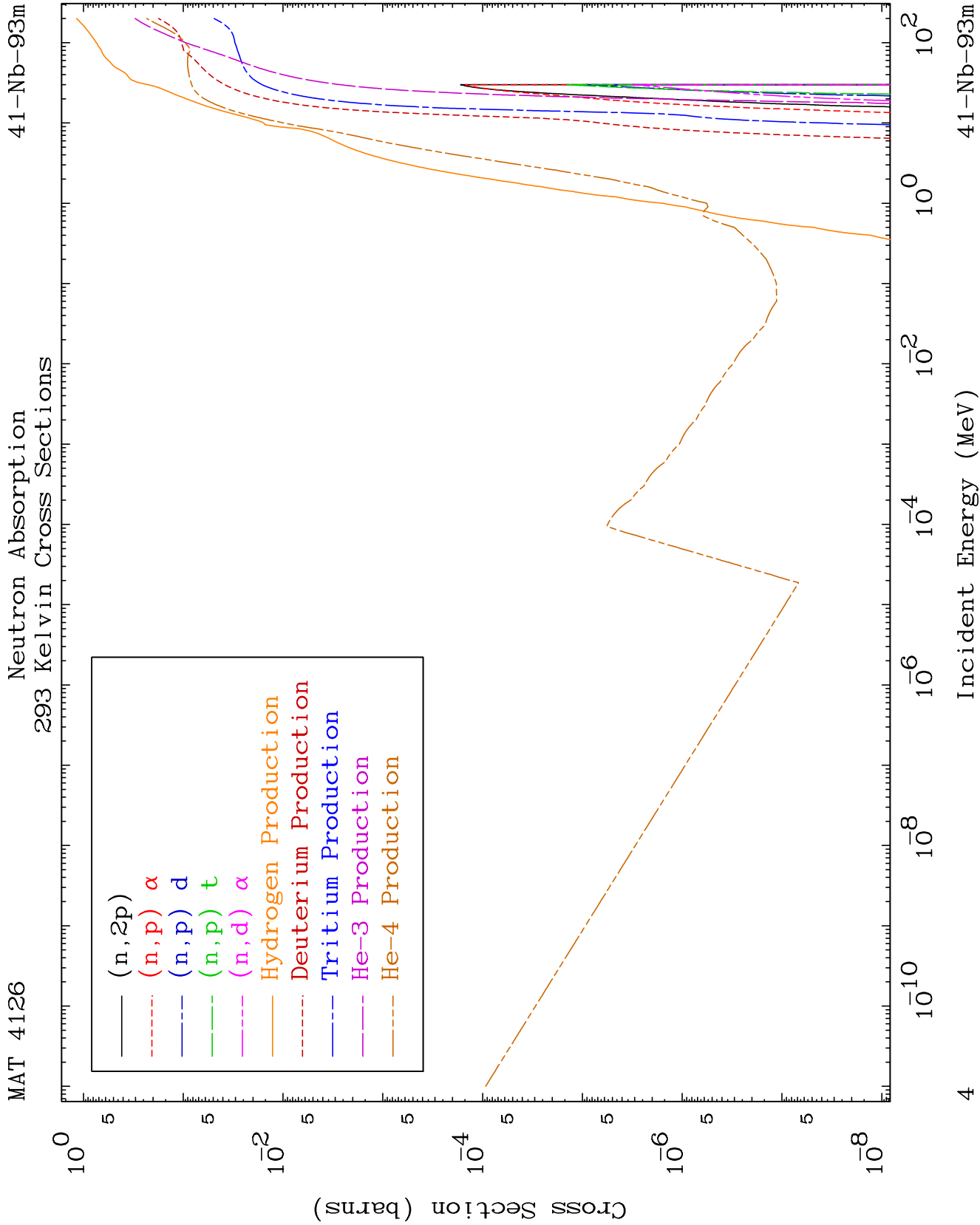
MAT 4126

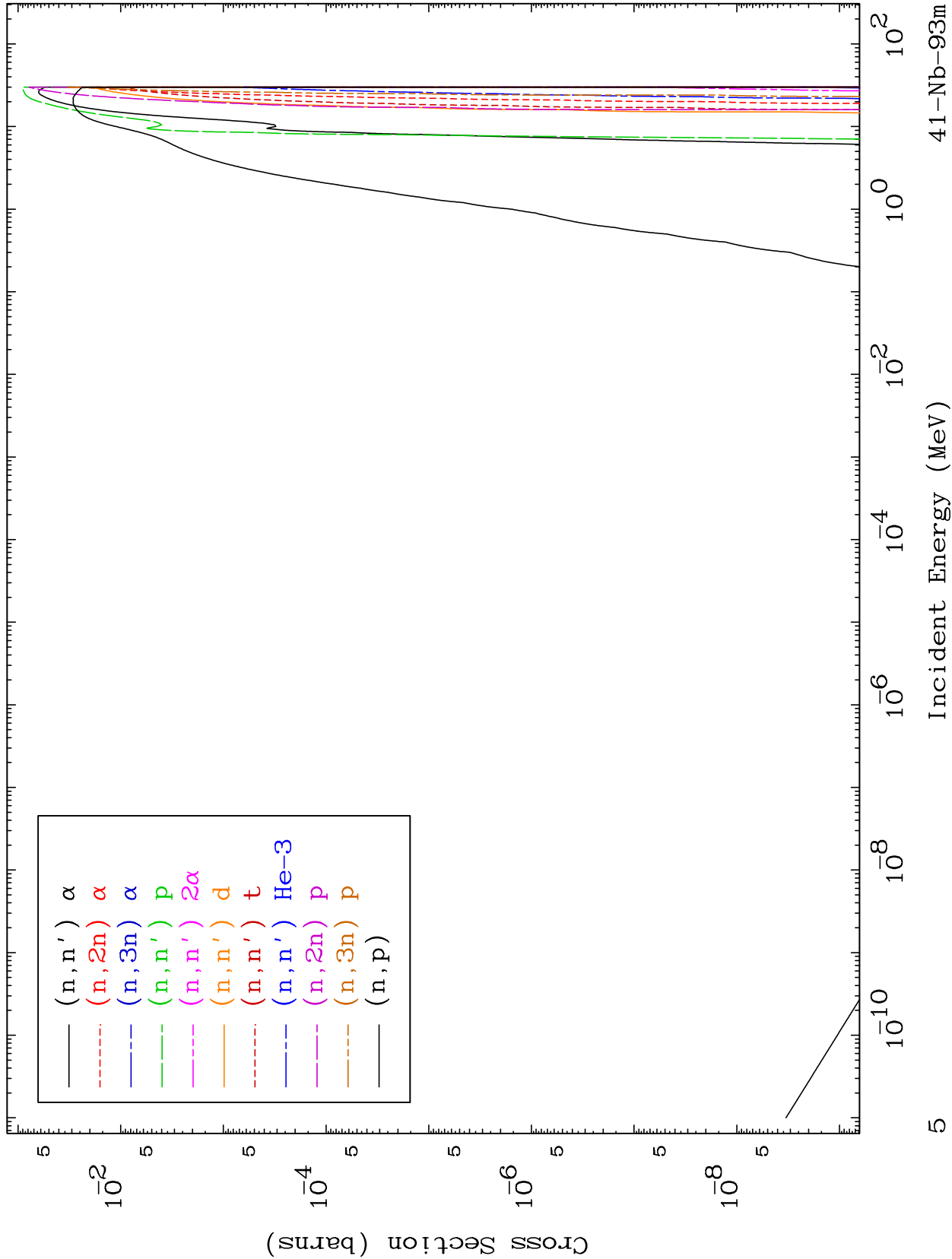
Neutron Absorption
293 Kelvin Cross Sections

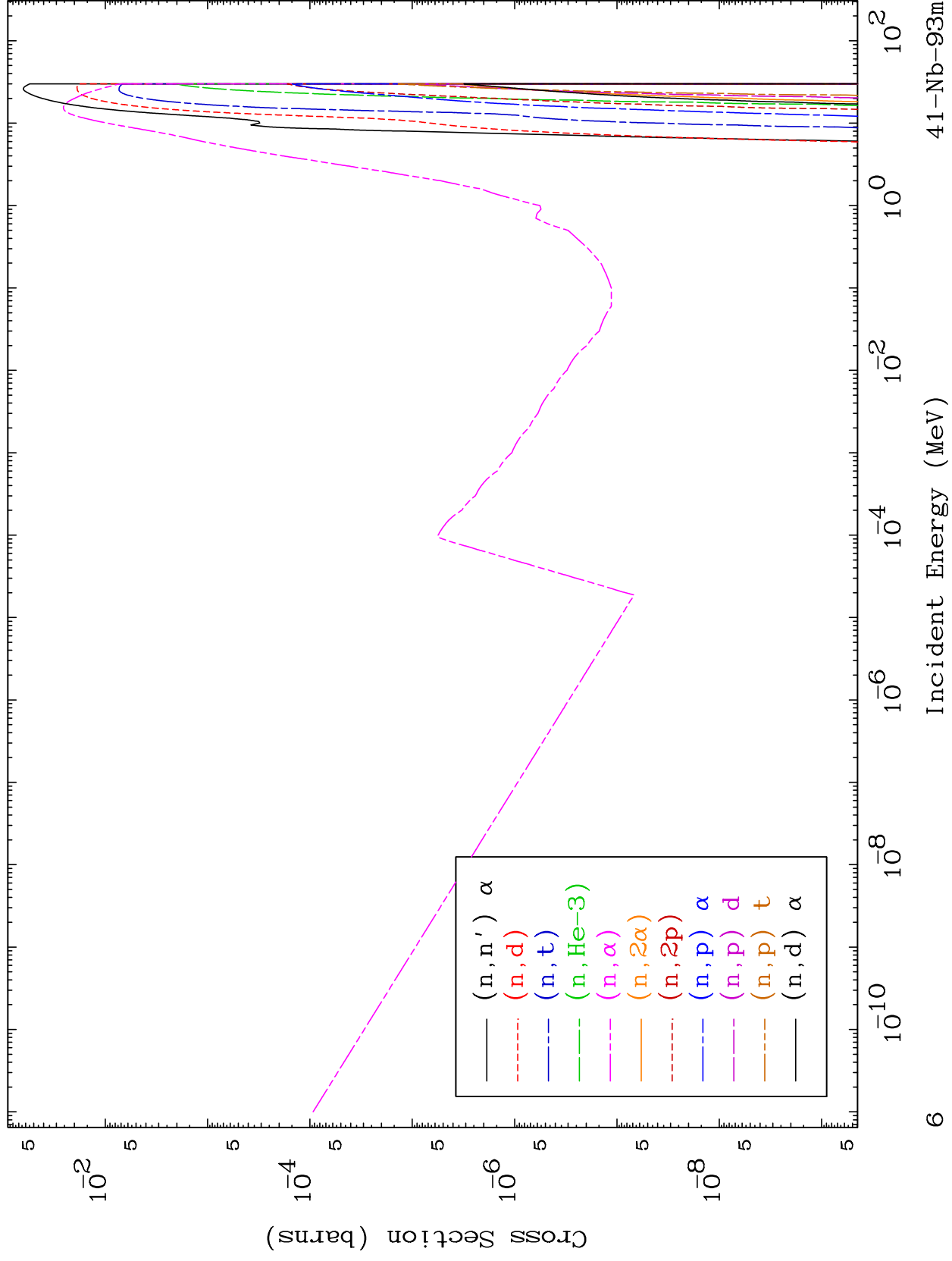
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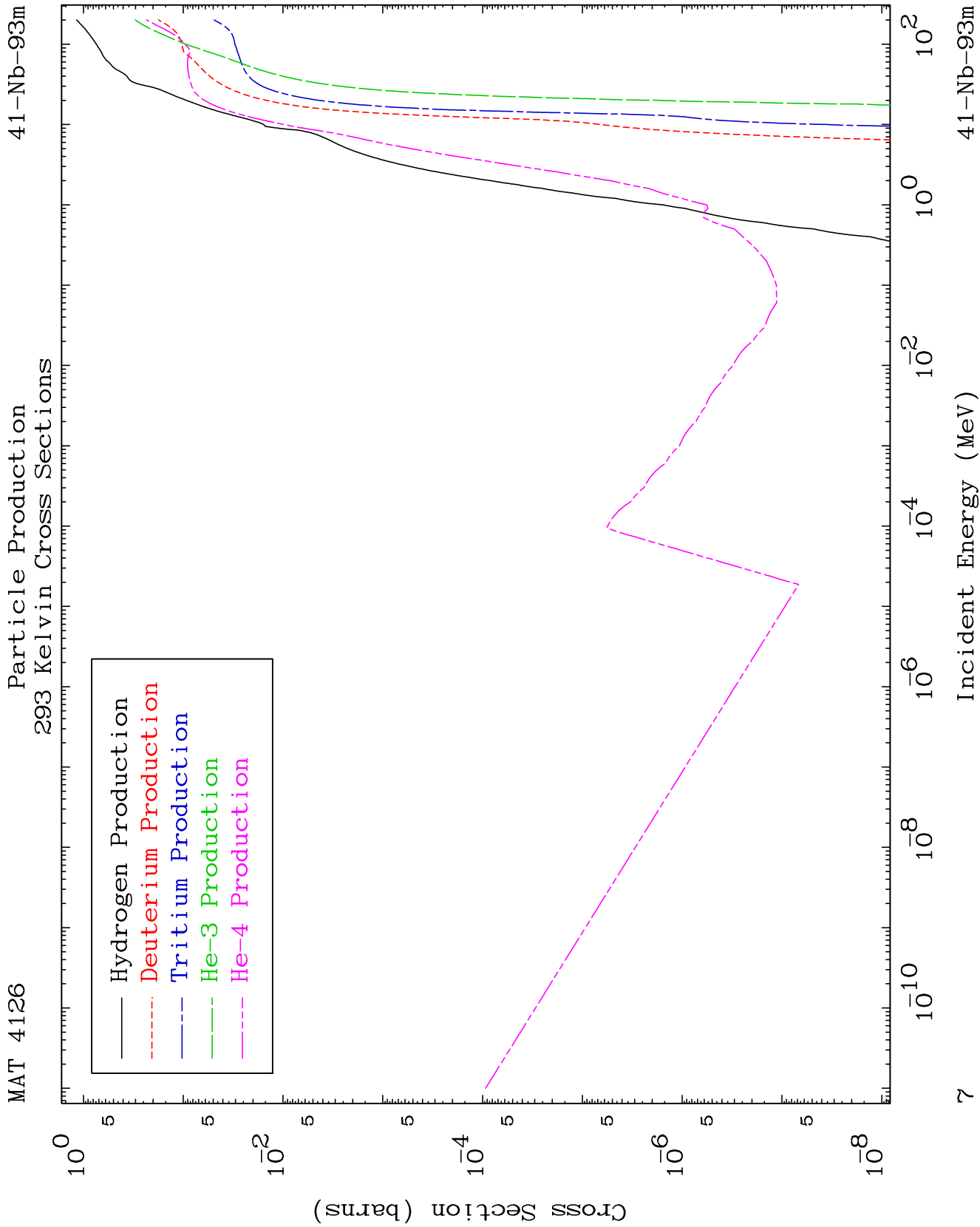


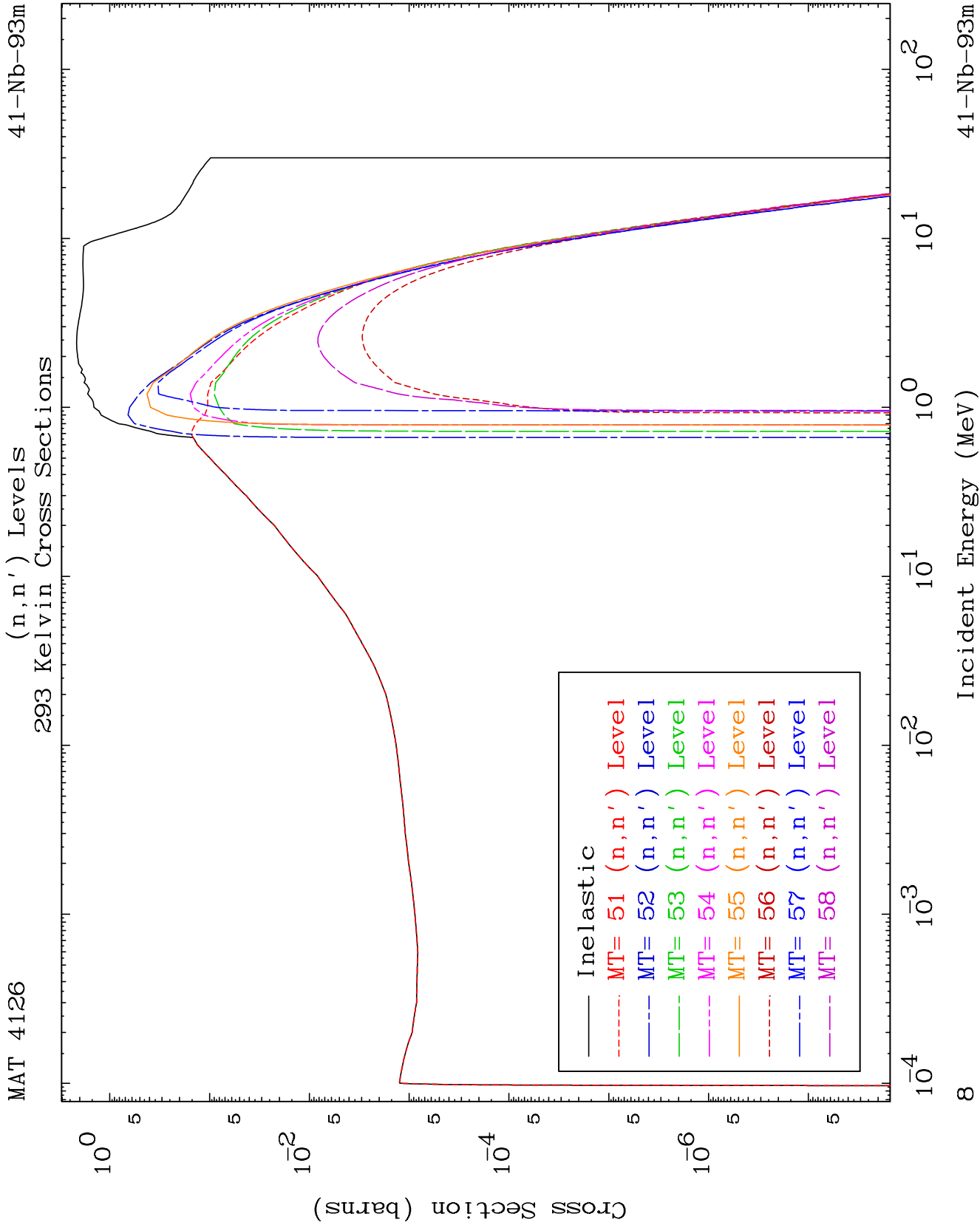


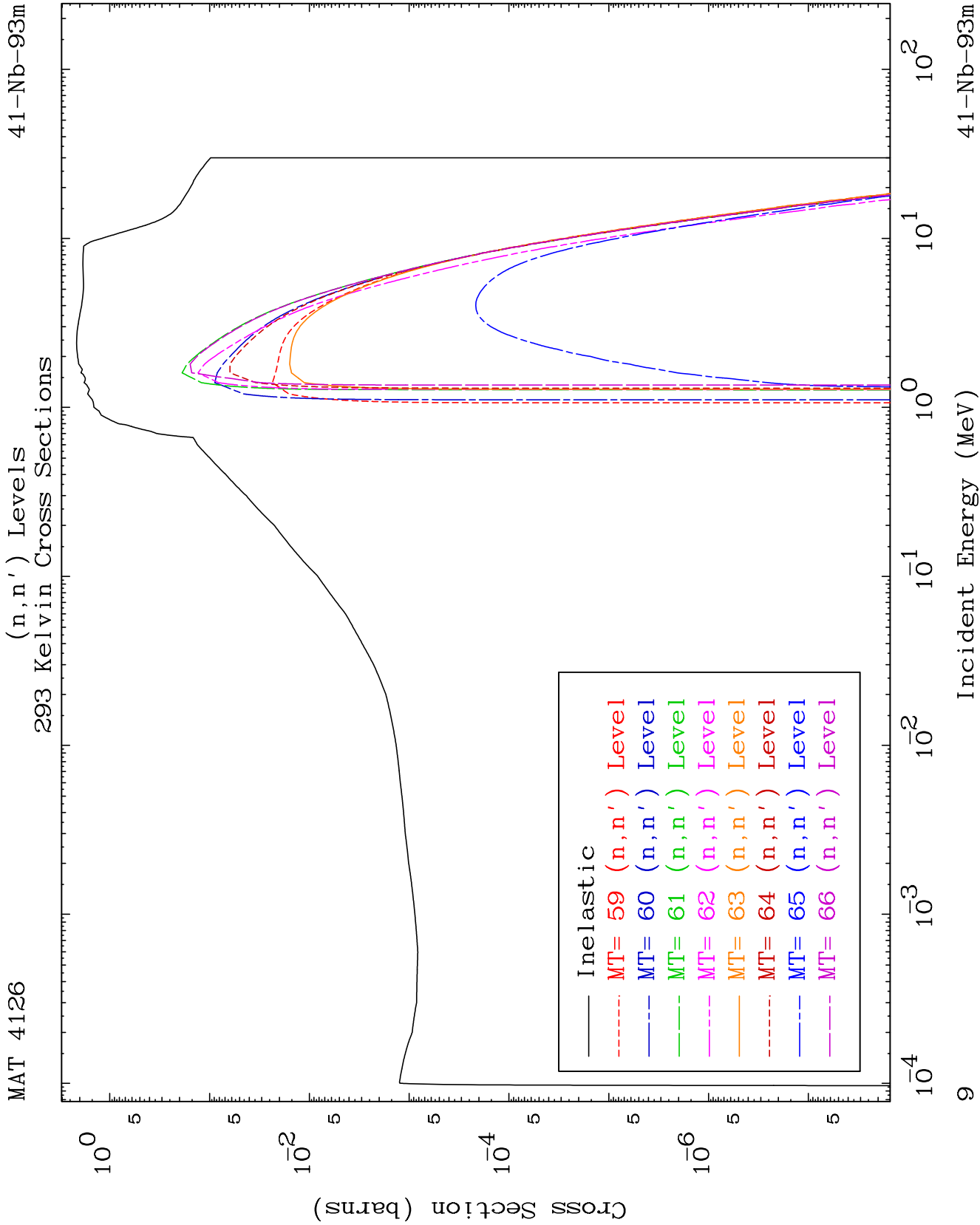


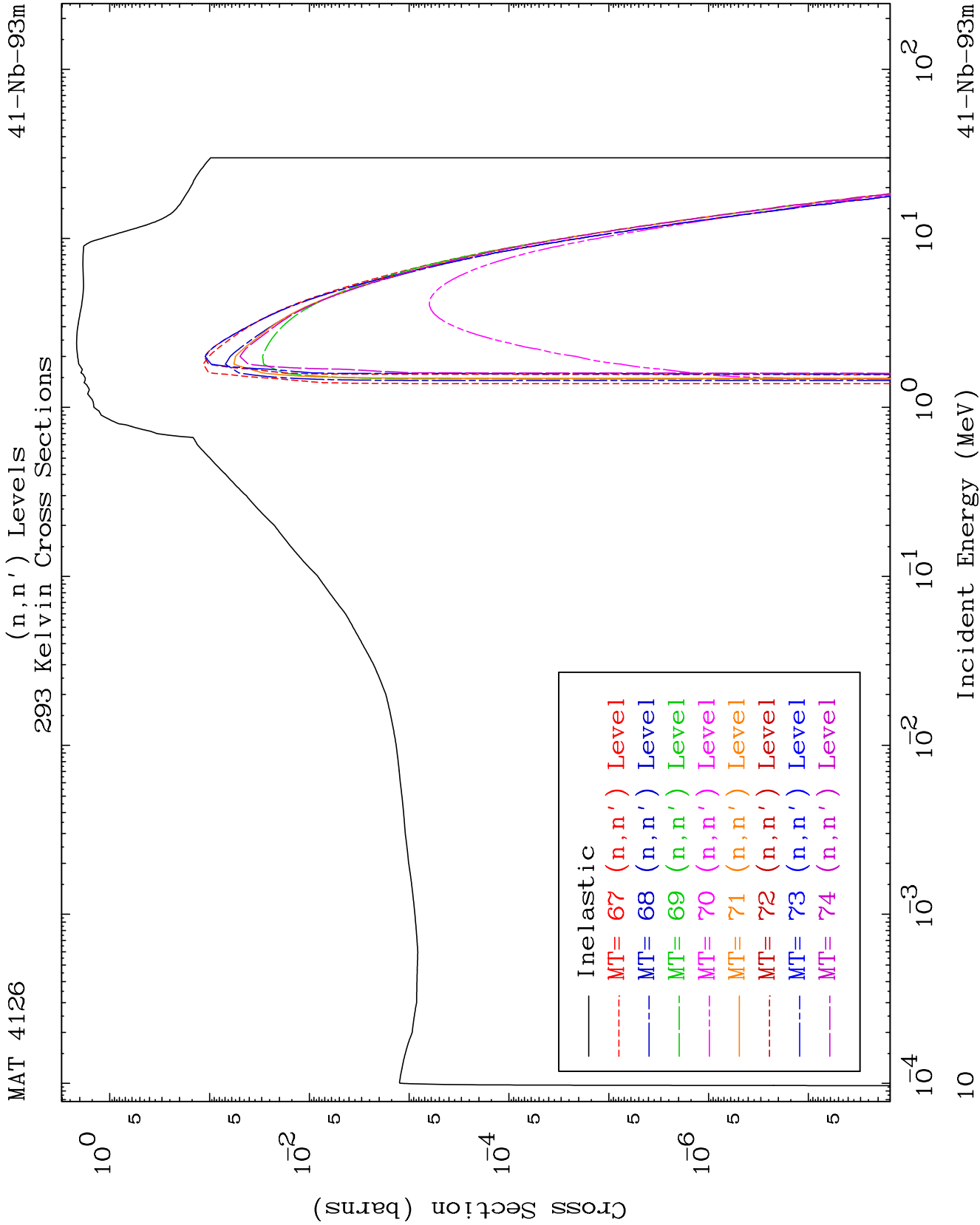








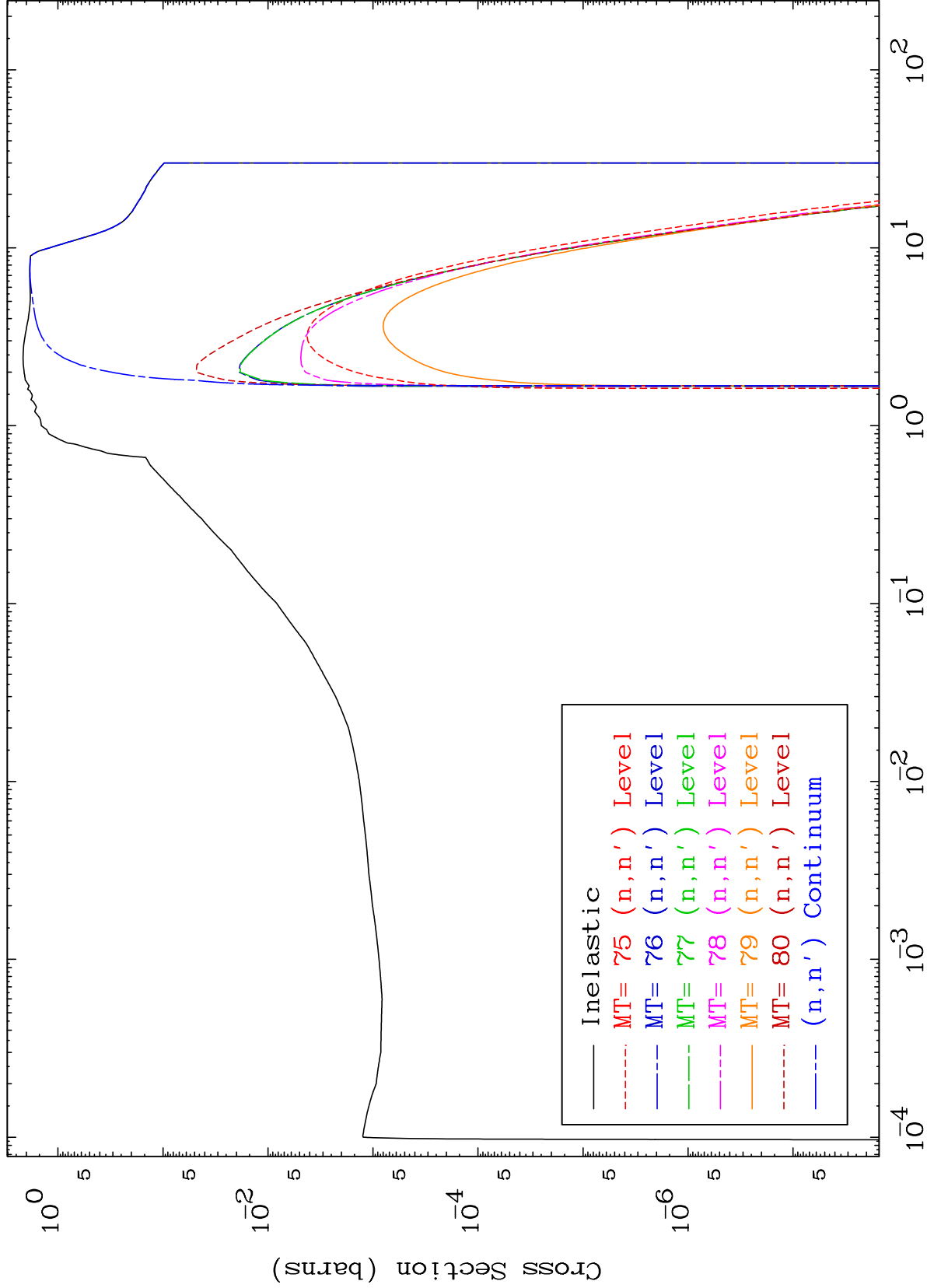




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(n,n') Levels
293 Kelvin Cross Sections

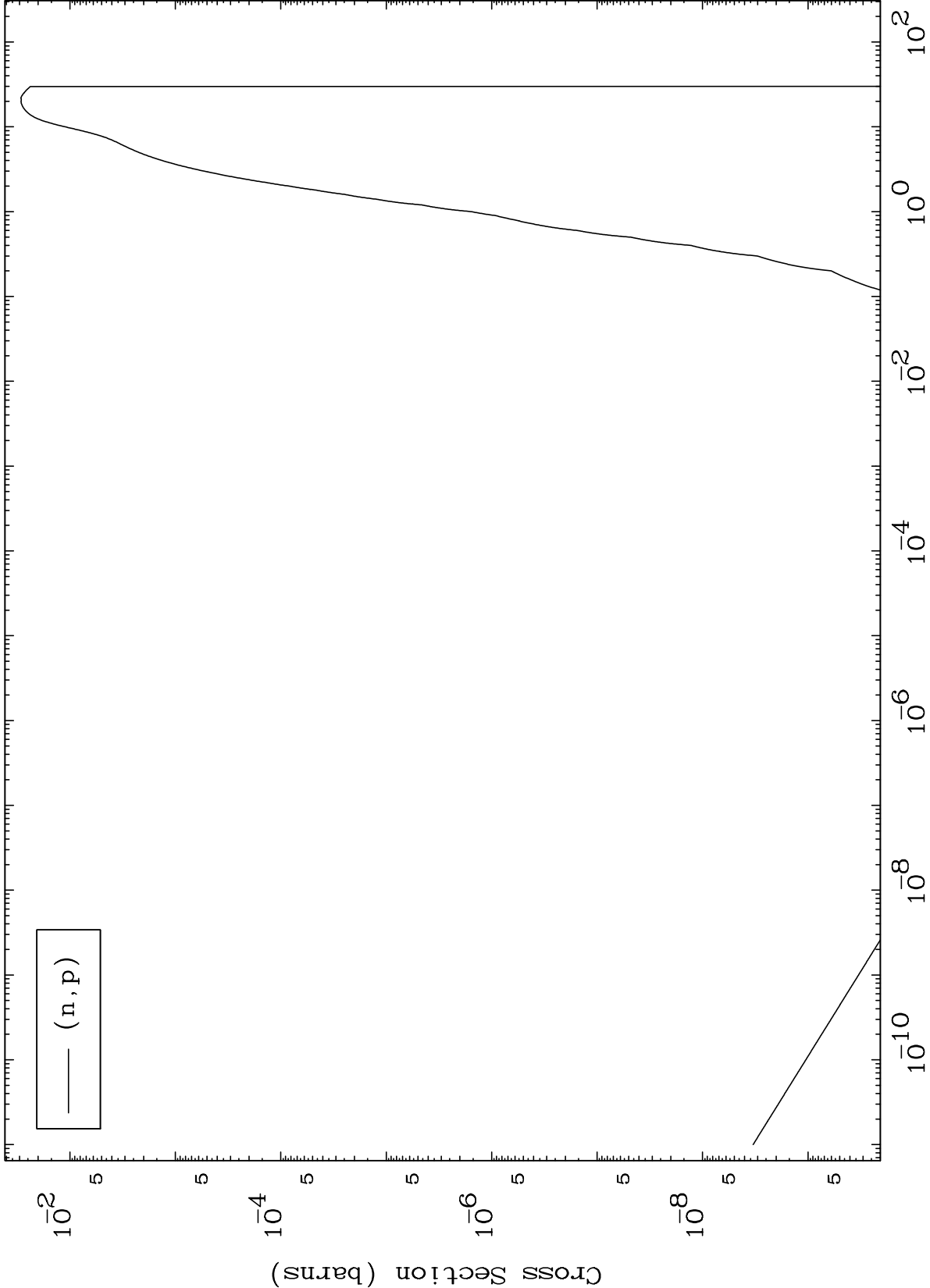
41-Nb-93m



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(n,p) Levels
293 Kelvin Cross Sections

41-Nb-93m



12

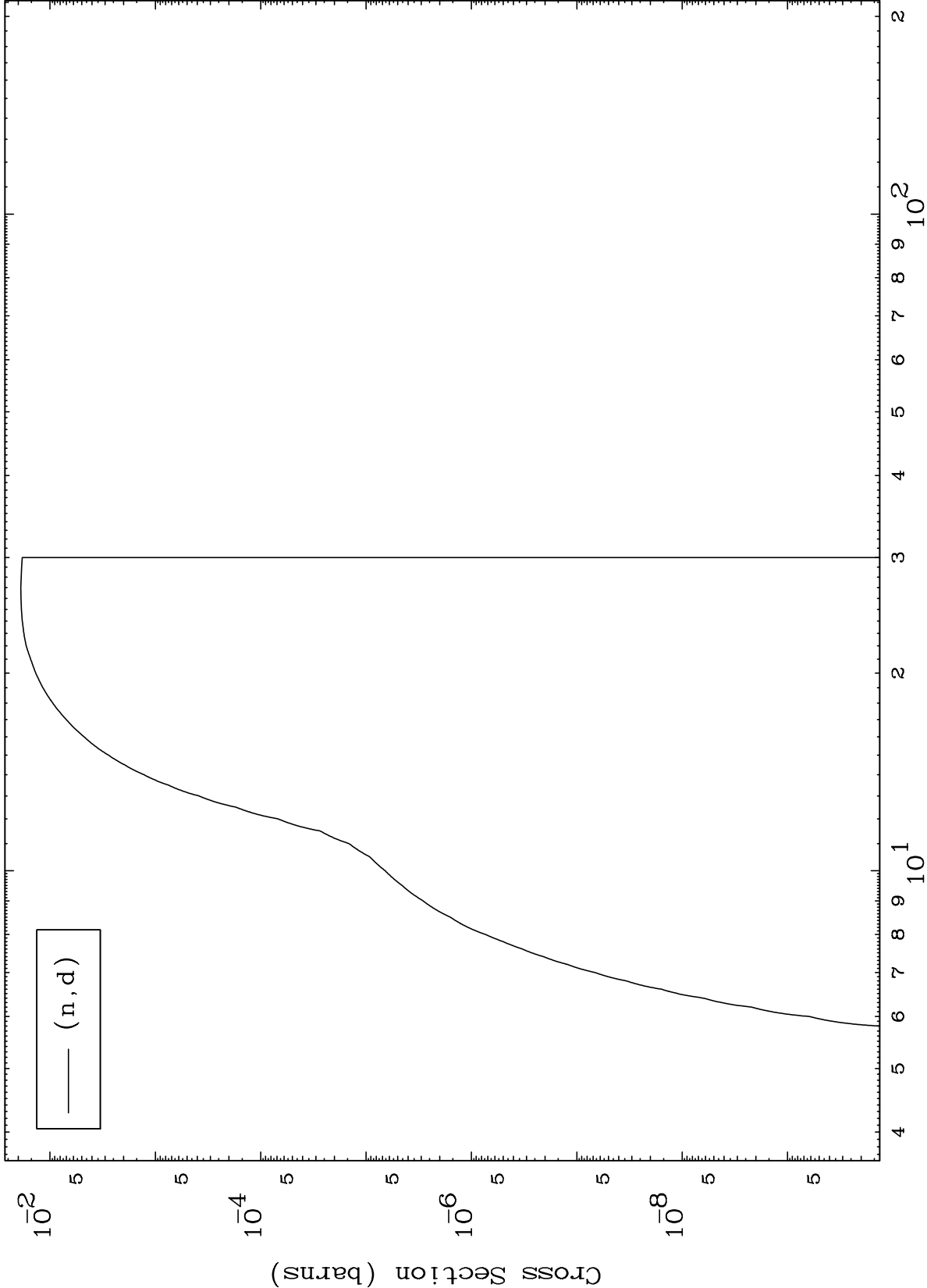
Incident Energy (MeV)

41-Nb-93m

MAT 4126

(n,d) Levels
293 Kelvin Cross Sections

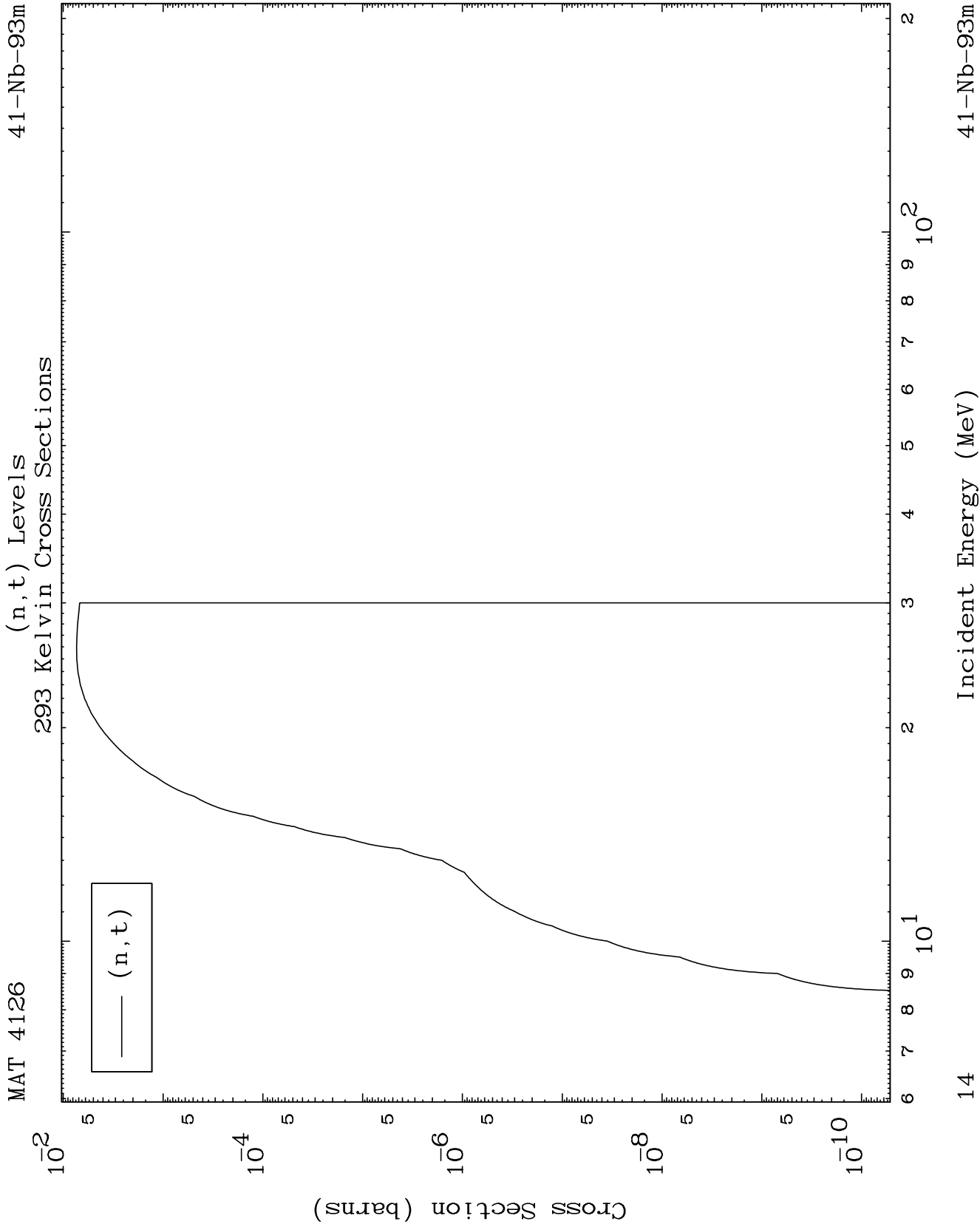
41-Nb-93m



Incident Energy (MeV)

41-Nb-93m

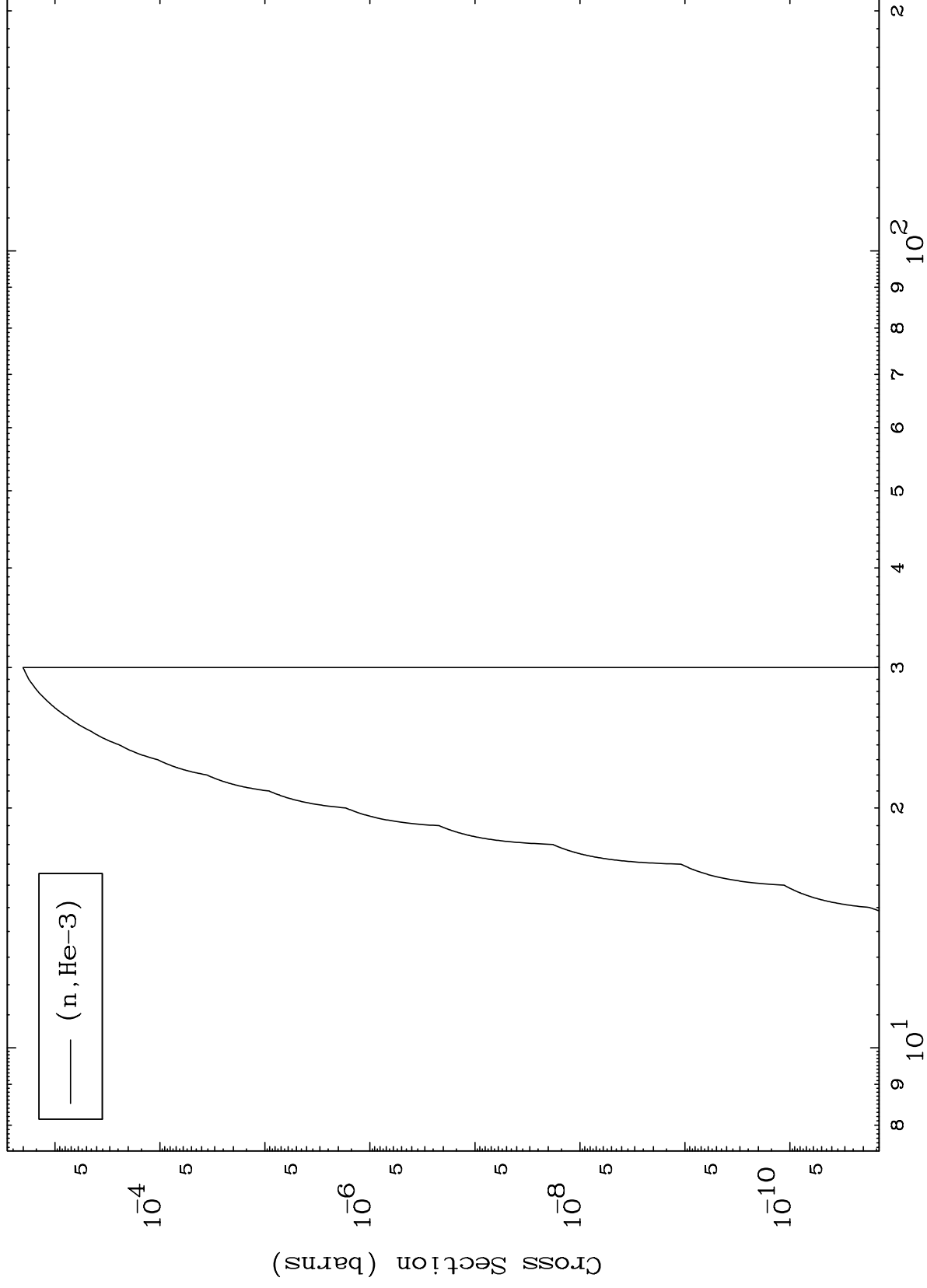
13



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(n,He3) Levels
293 Kelvin Cross Sections

41-Nb-93m



15

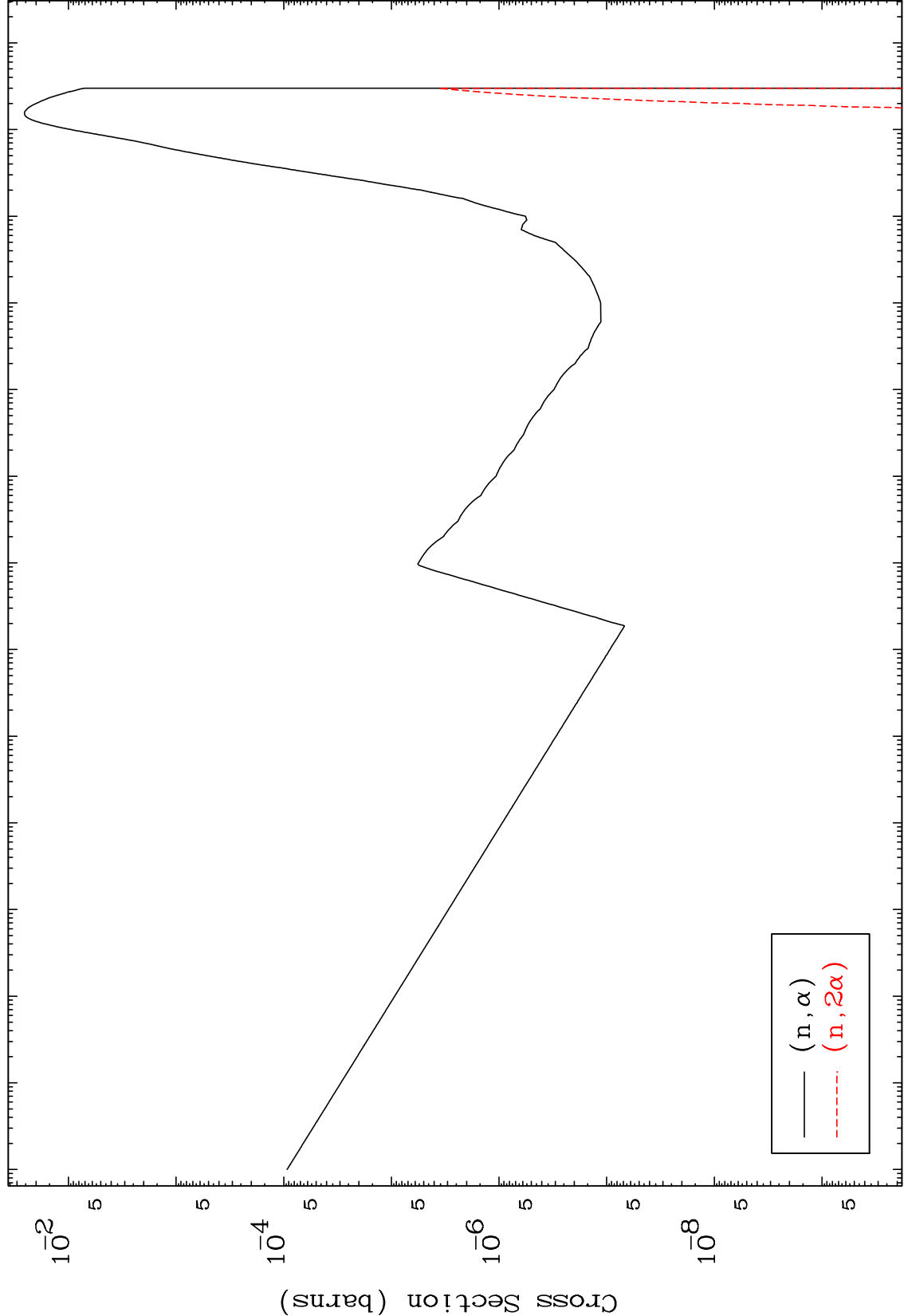
Incident Energy (MeV)

41-Nb-93m

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(n, α) Levels
293 Kelvin Cross Sections

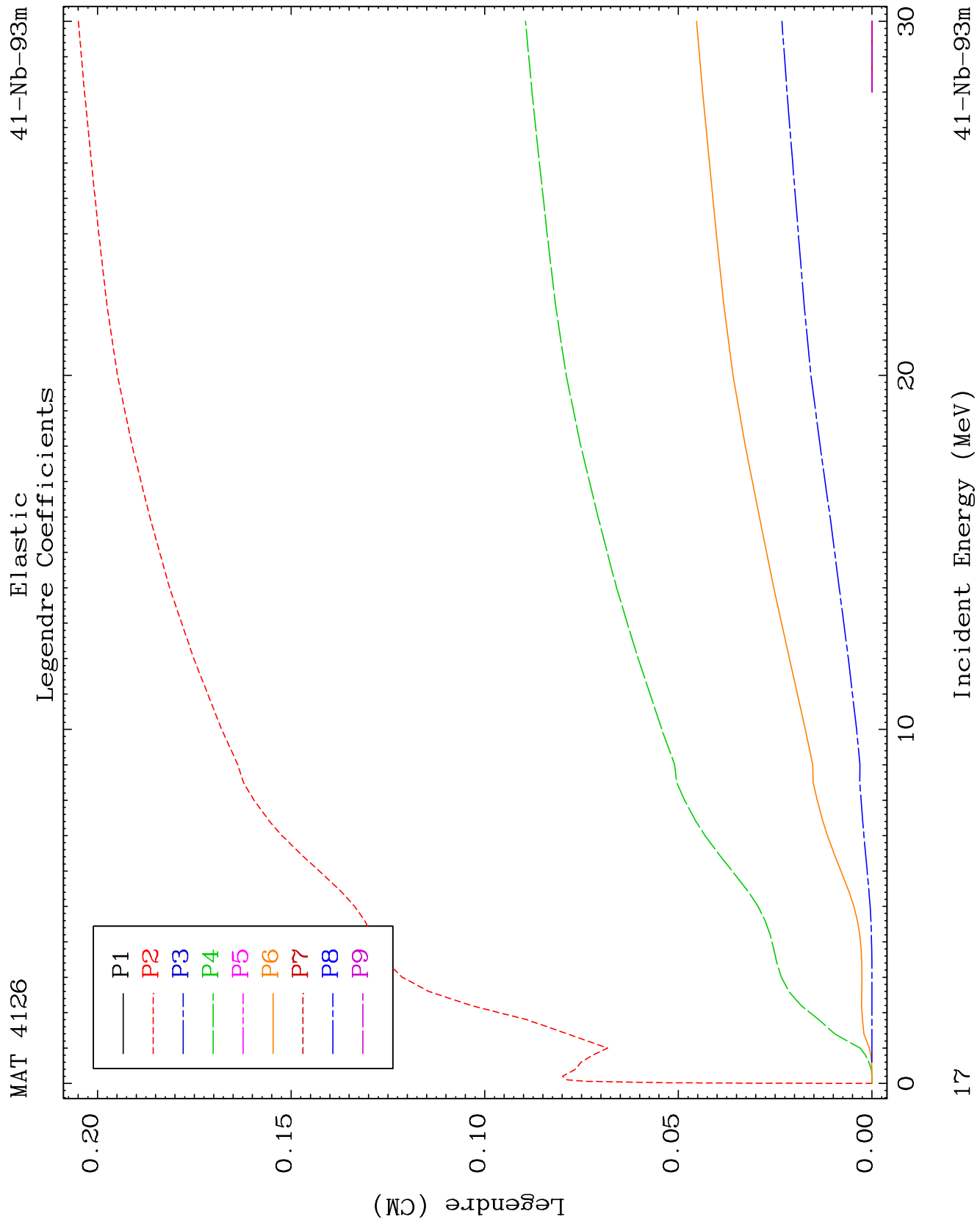
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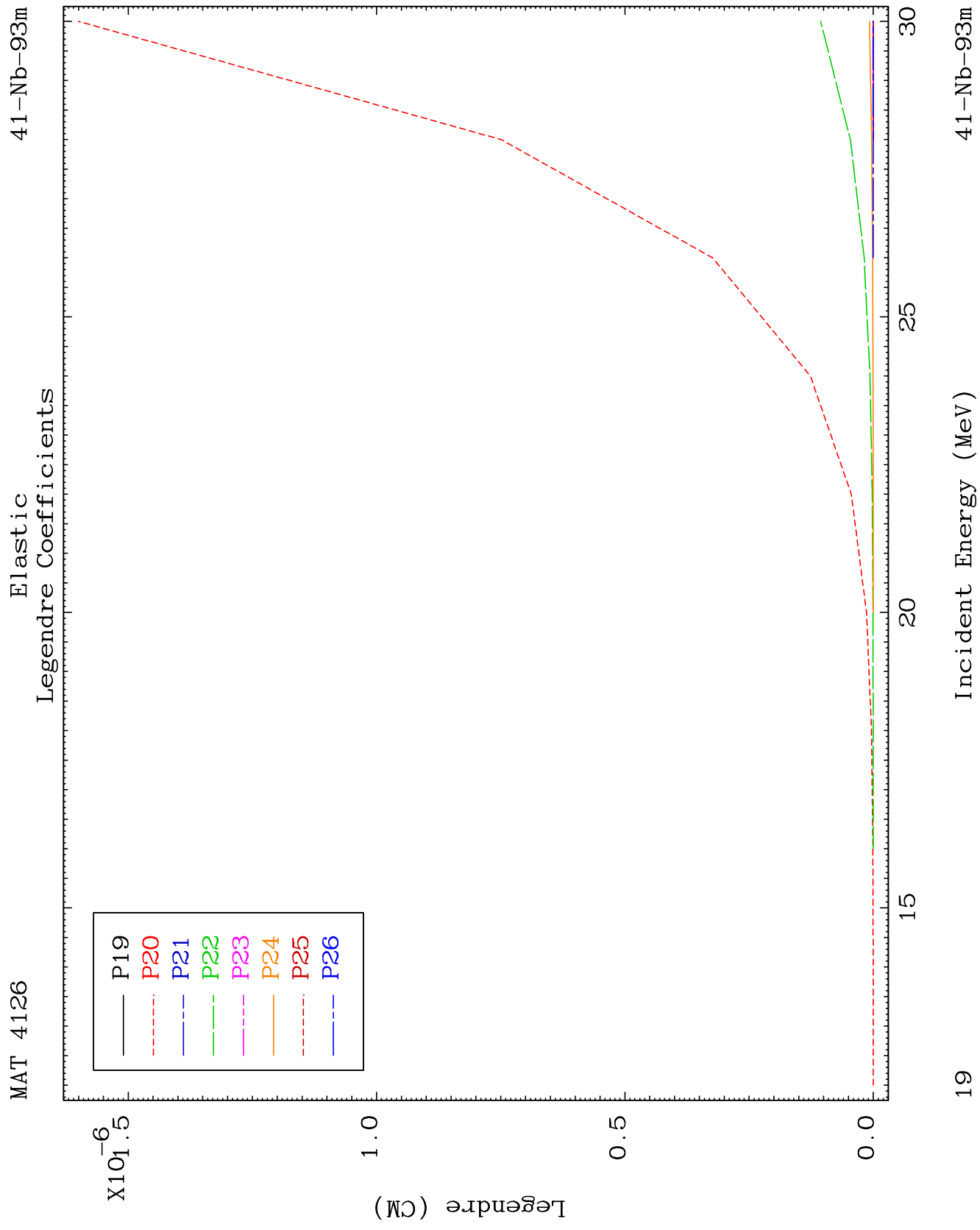


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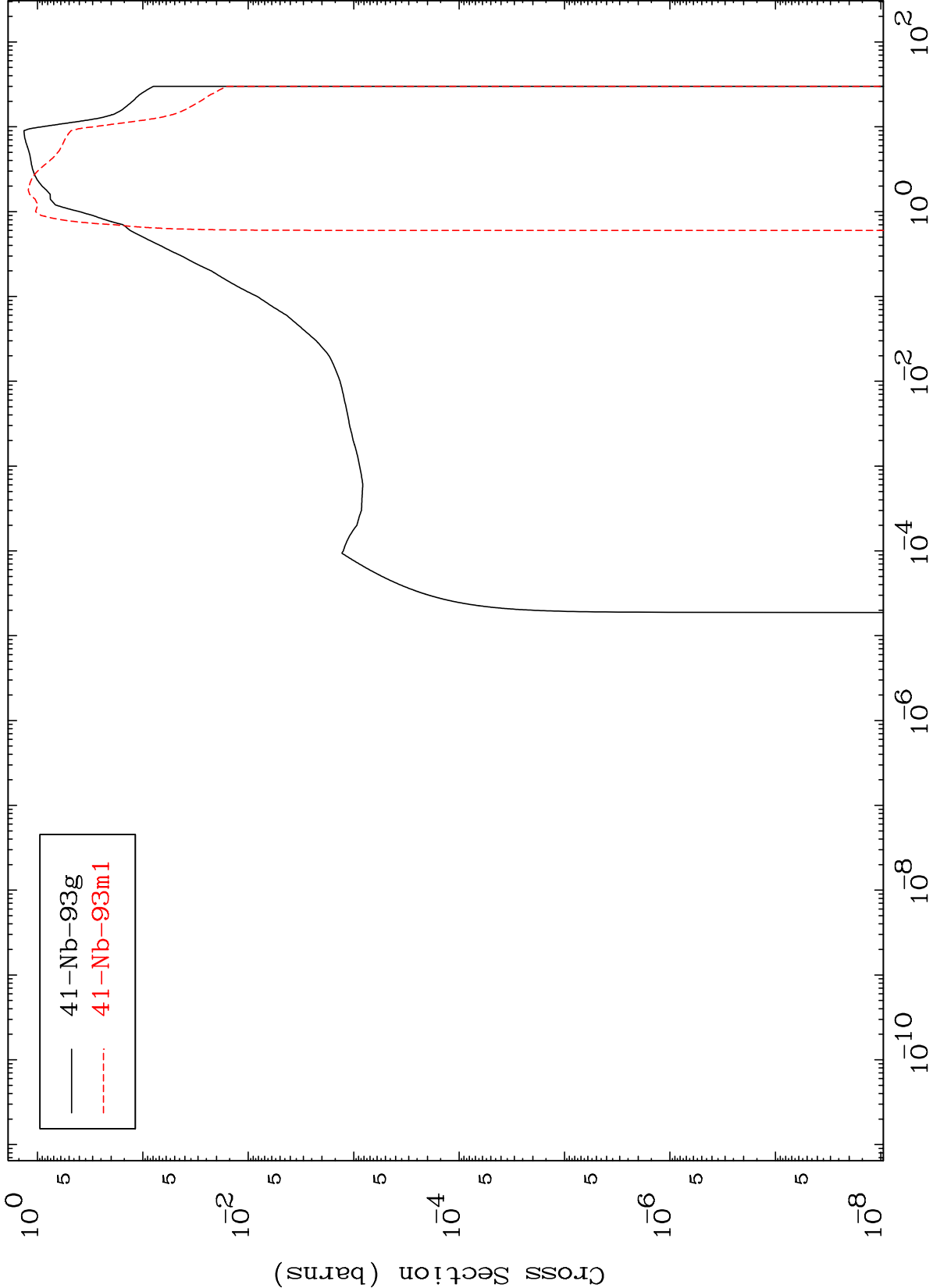
Incident Energy (MeV)

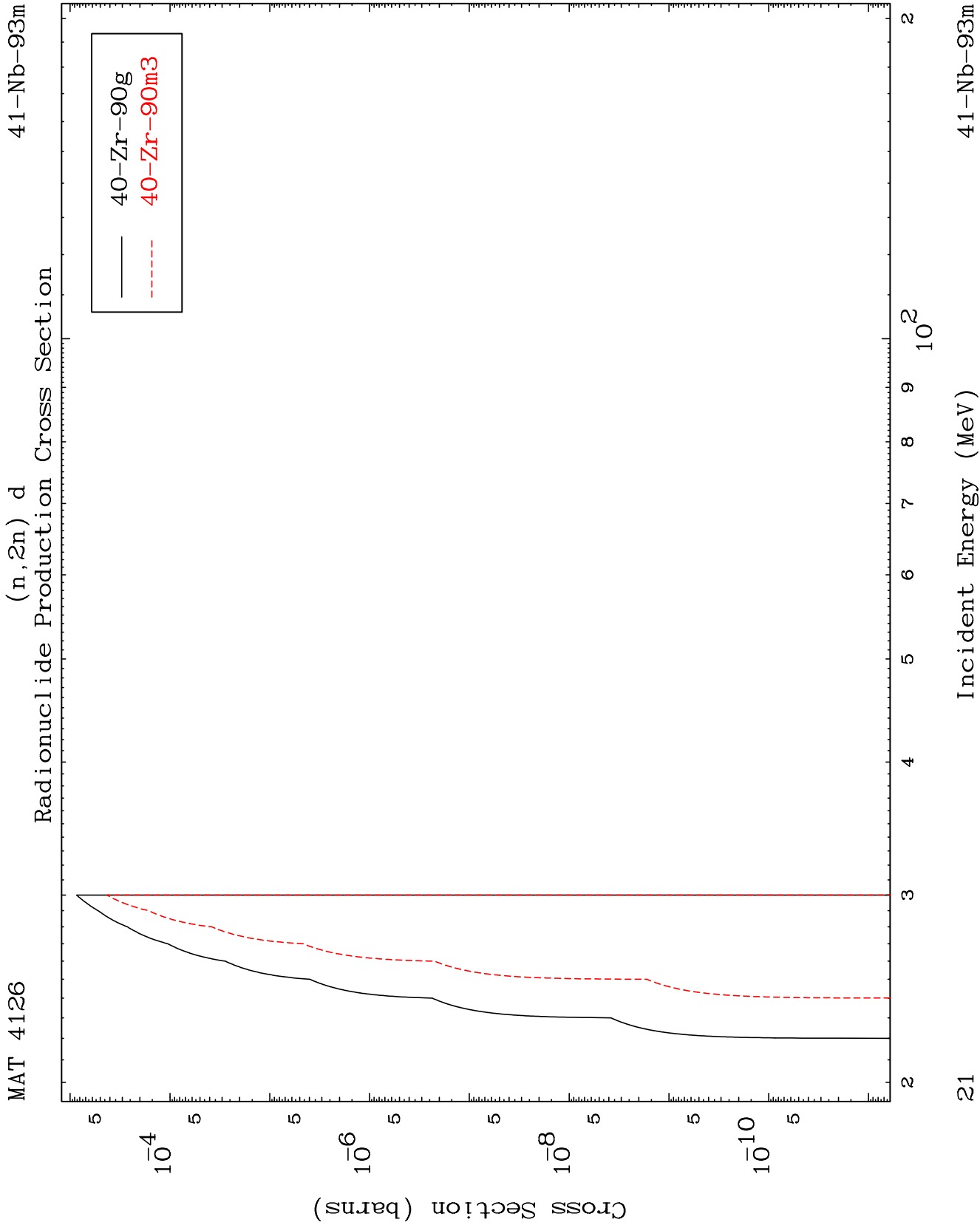
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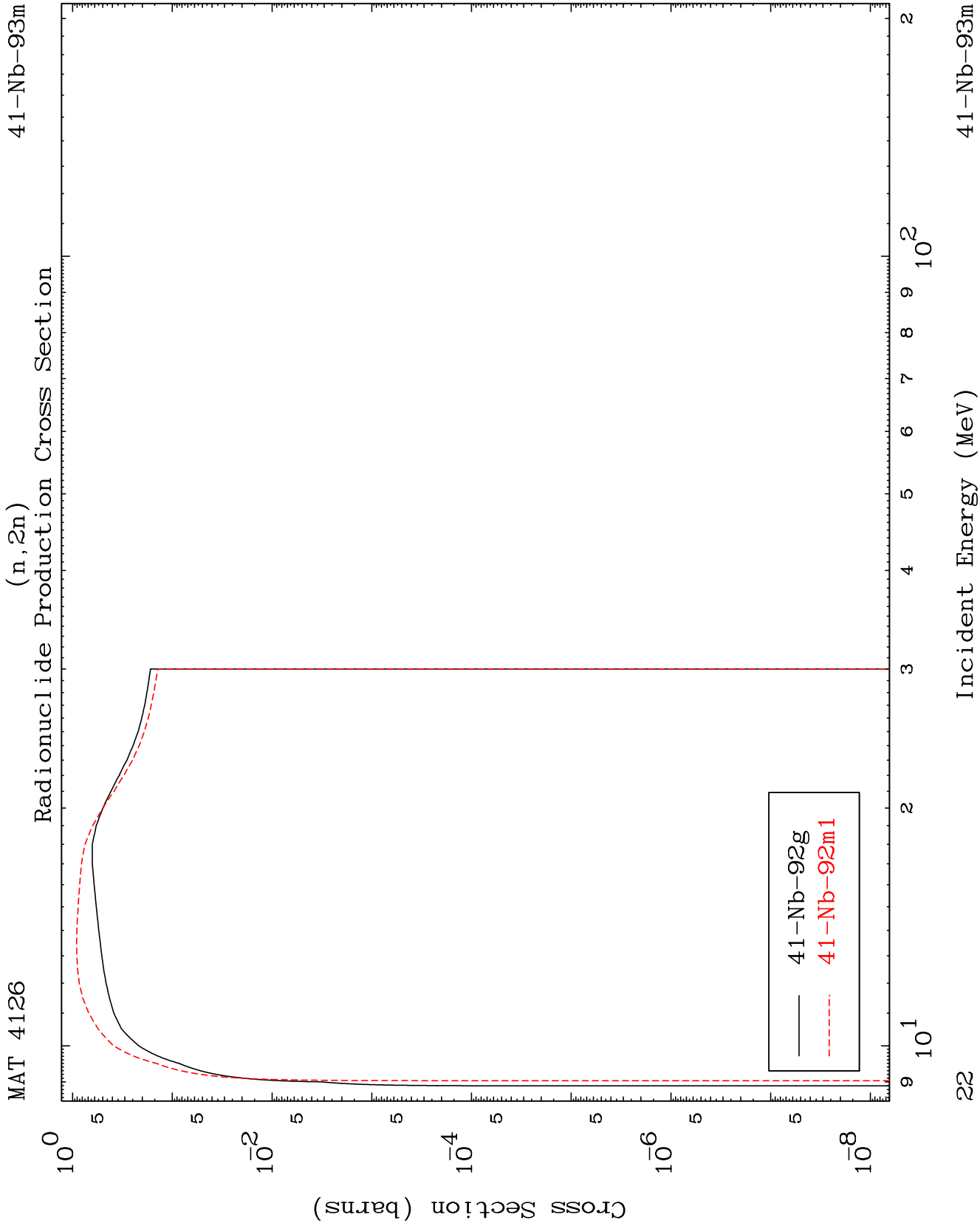


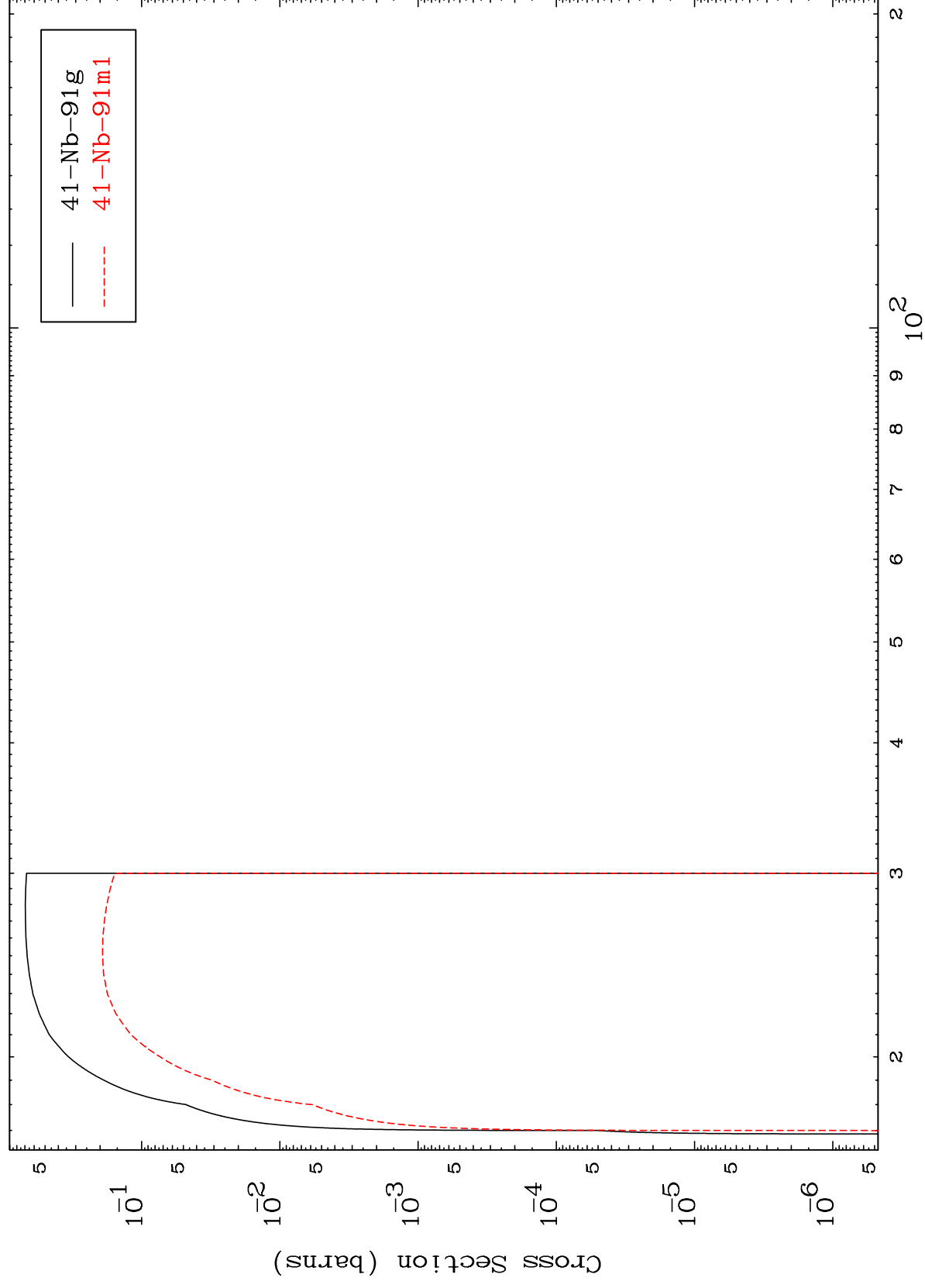


Inelastic
Radionuclide Production Cross Section







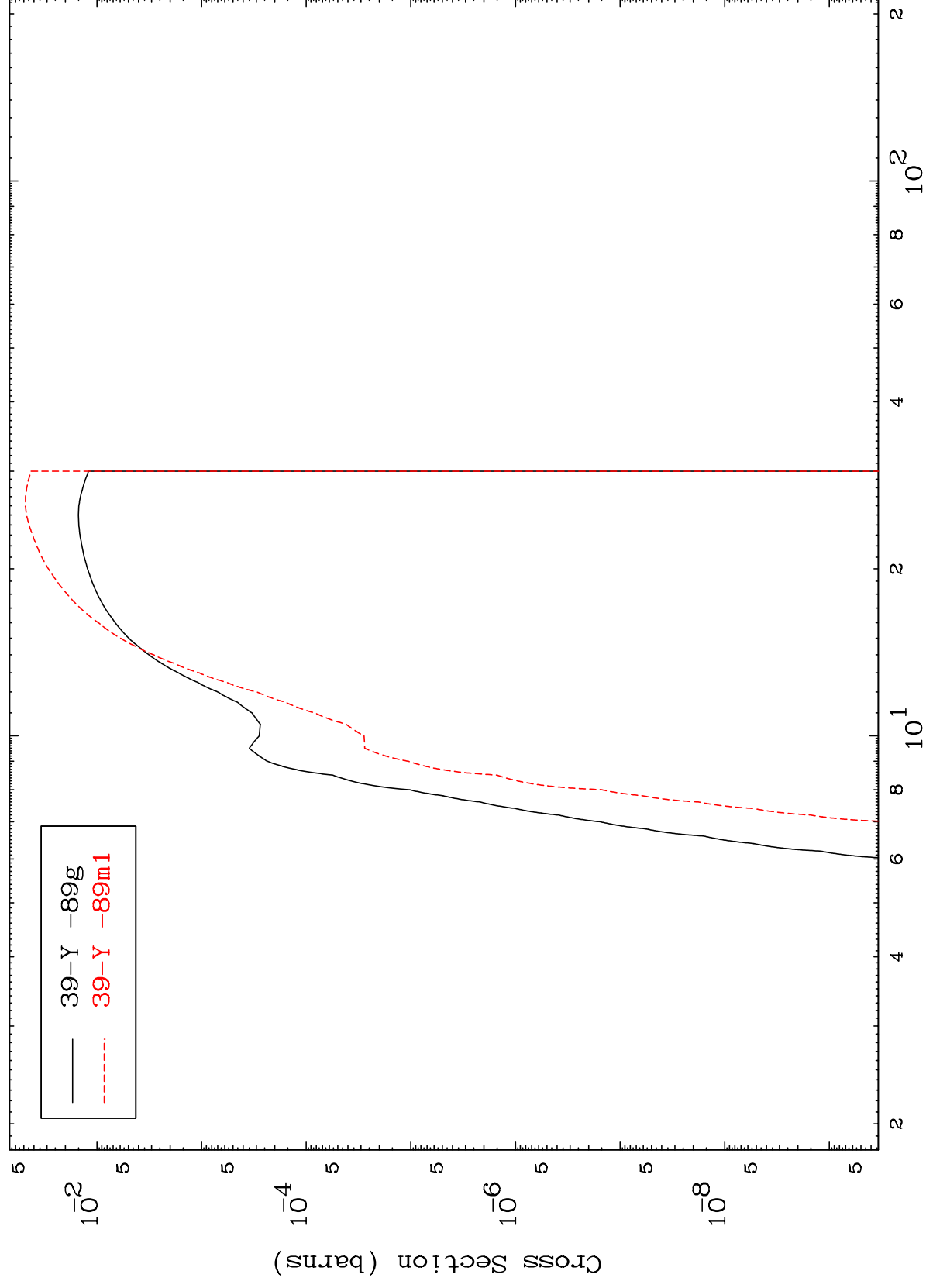


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41-Nb-93m

$$(n, n') \propto$$

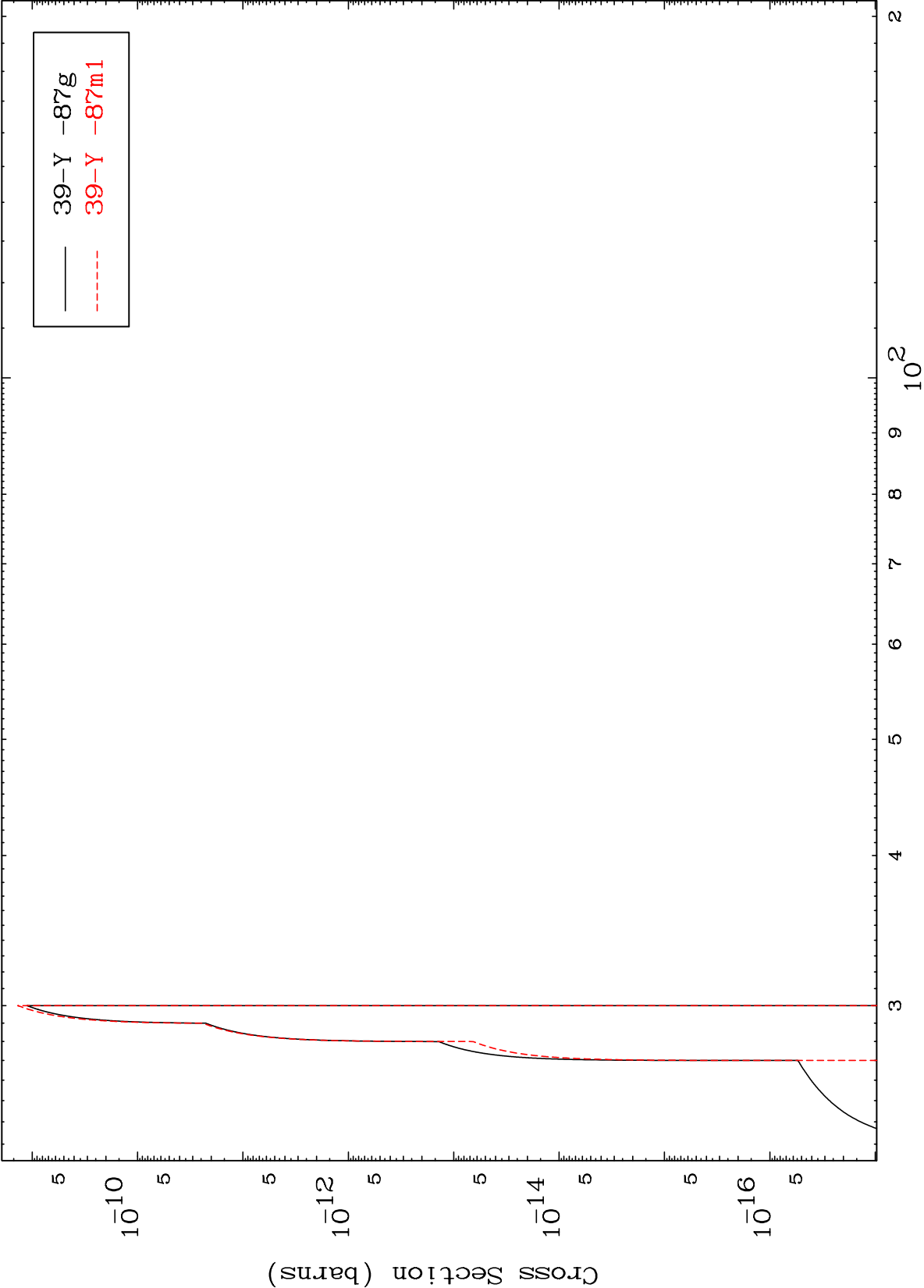
Radionuclide Production Cross Section



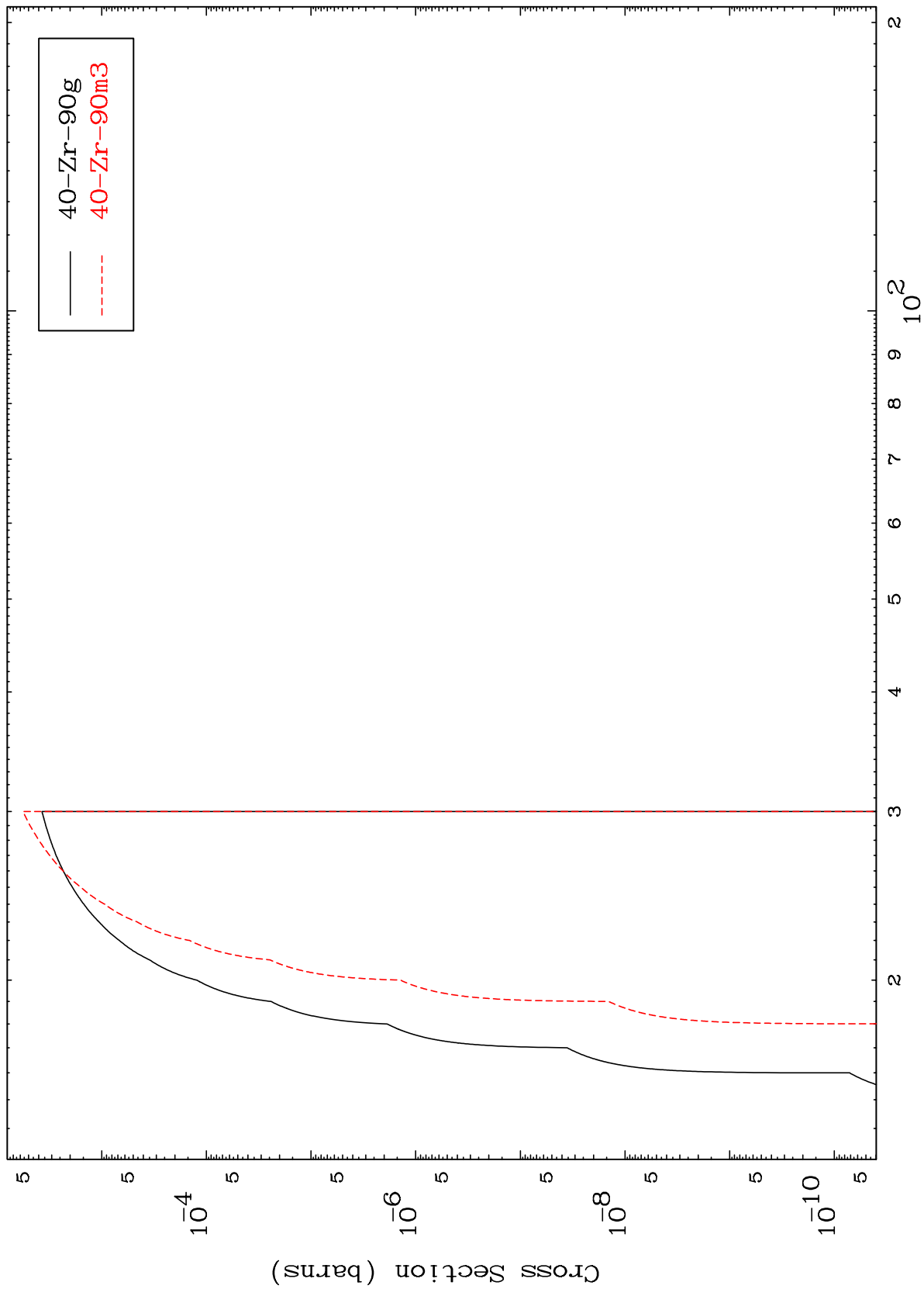
41-Nb-93m

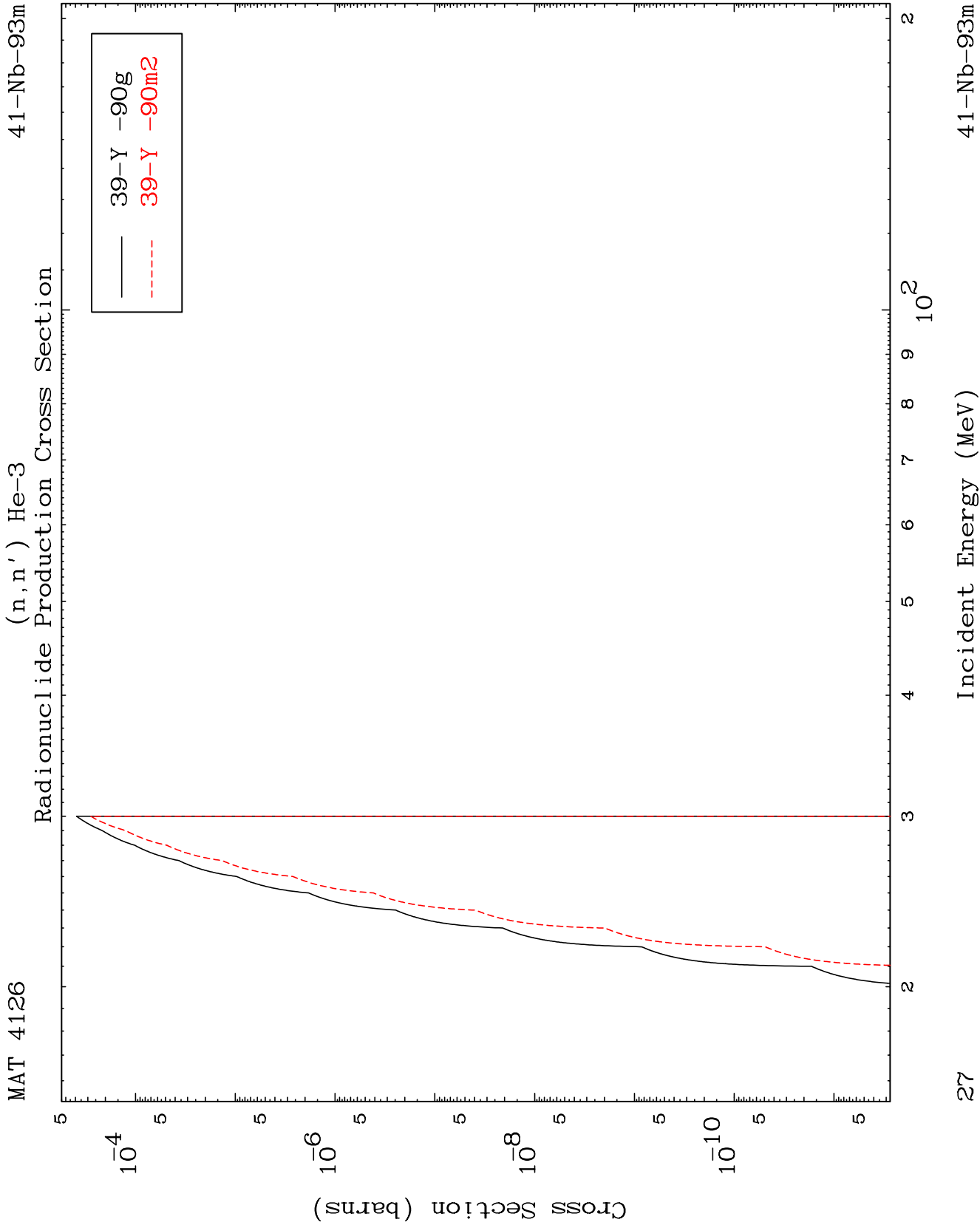
Incident Energy (MeV)

$(n,3n) \alpha$
Radionuclide Production Cross Section

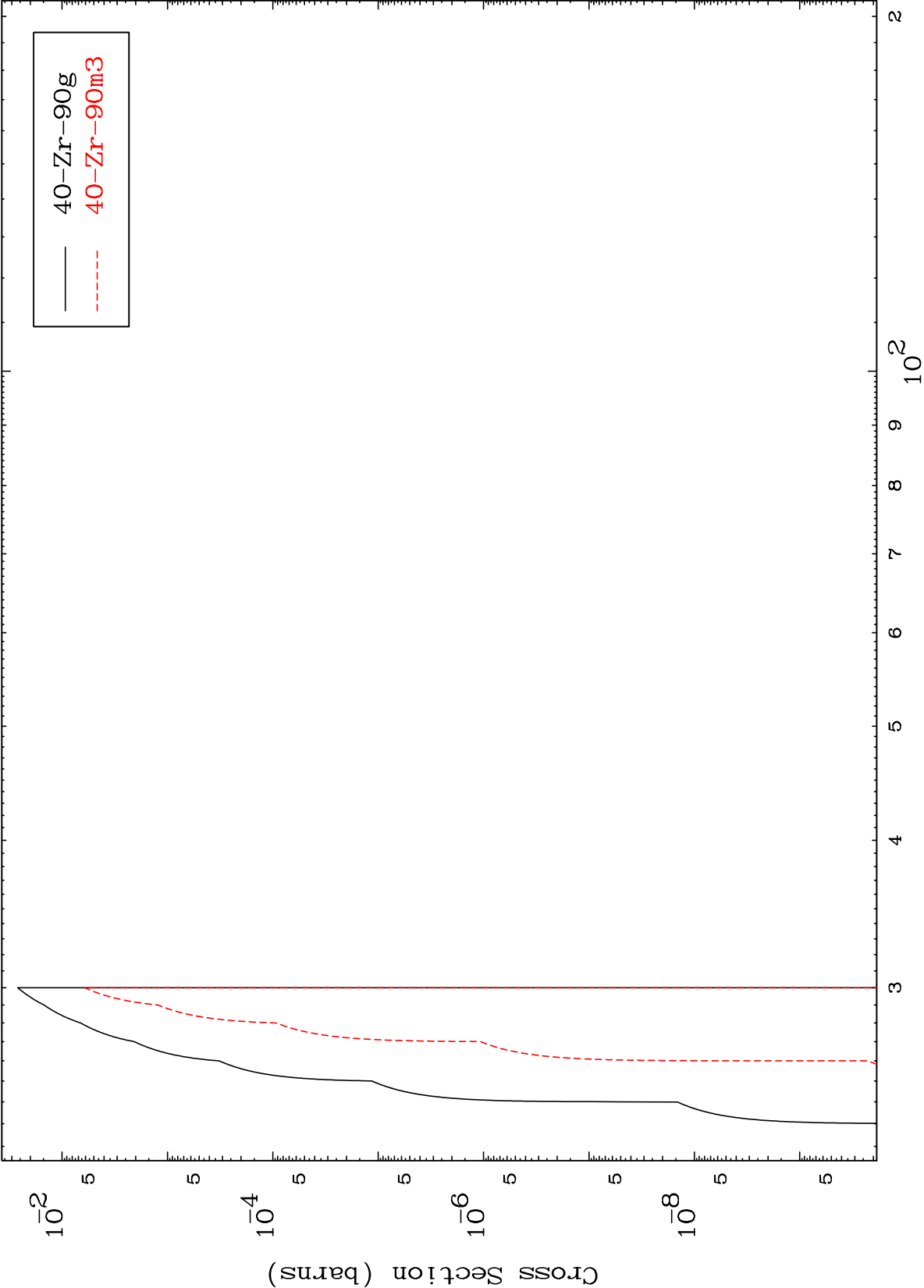


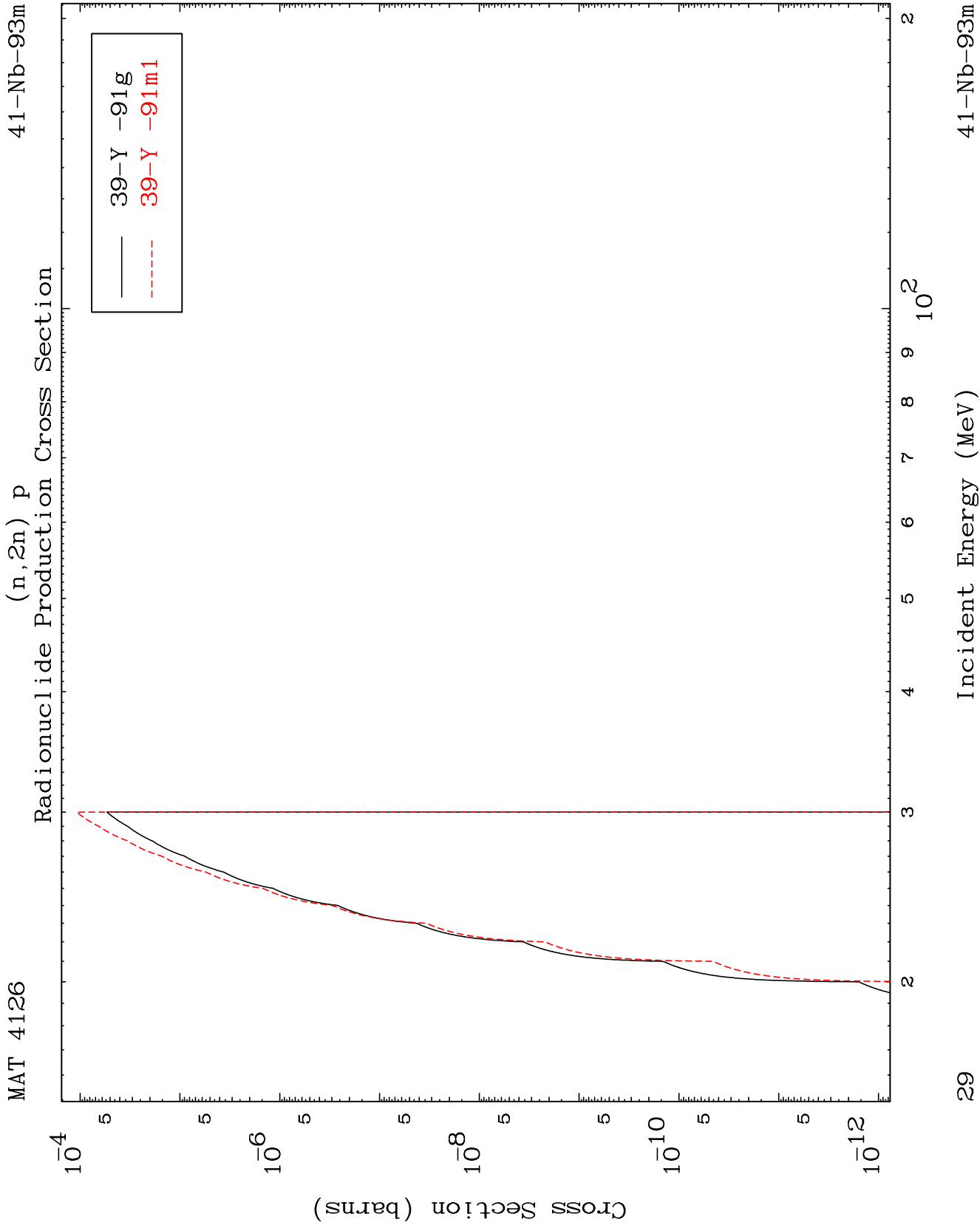
(n,n') t
Radionuclide Production Cross Section

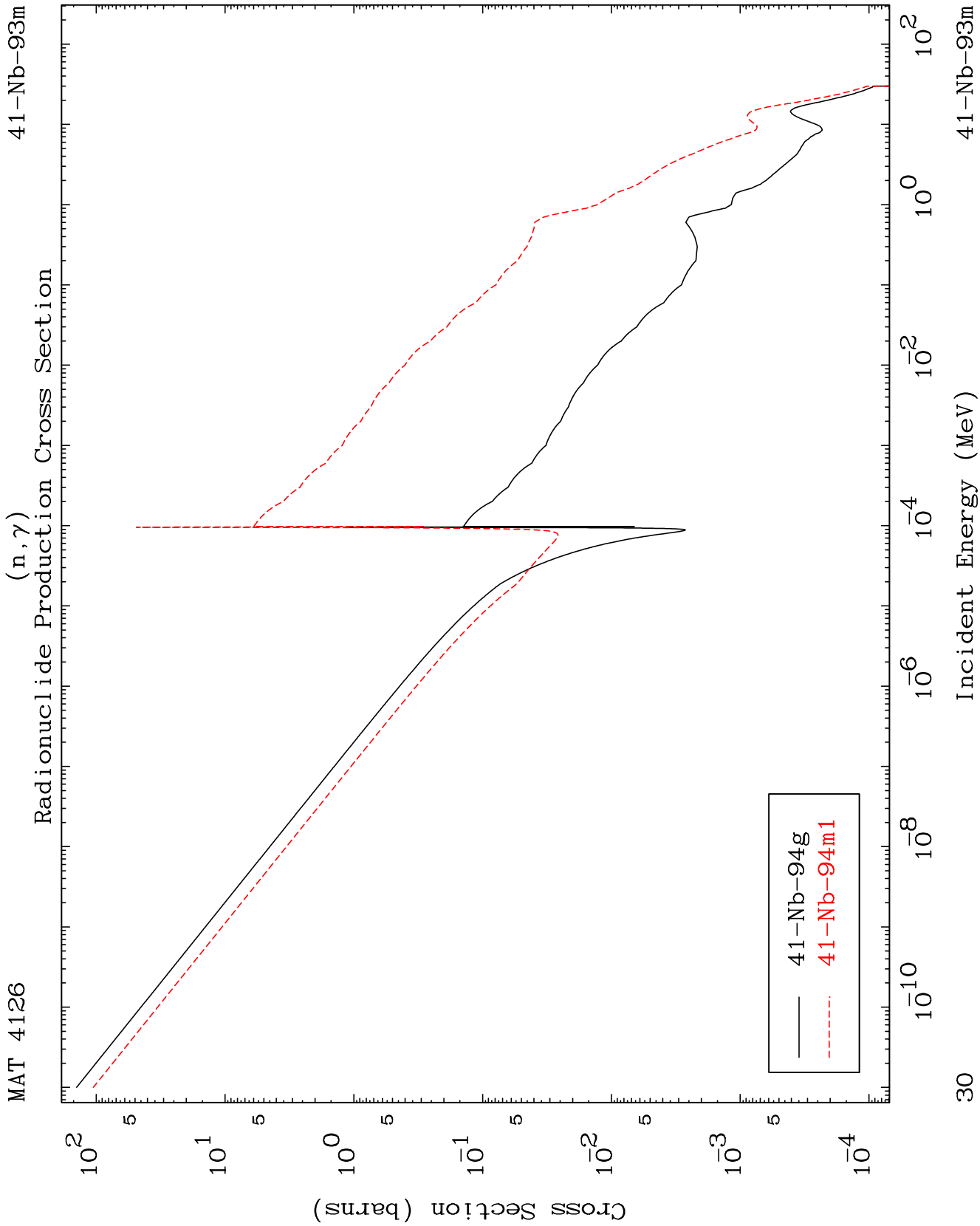


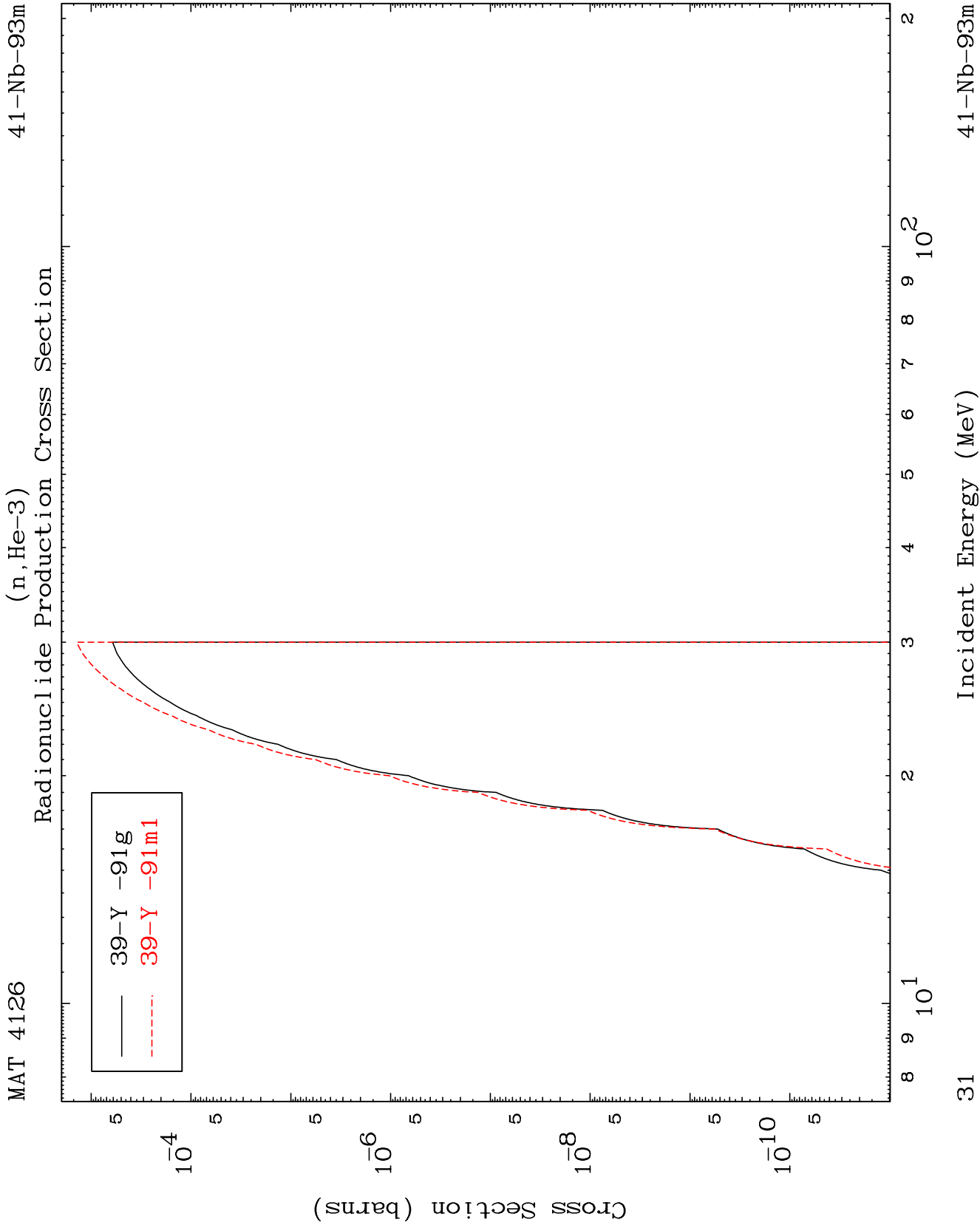


(n,3n) p
Radionuclide Production Cross Section





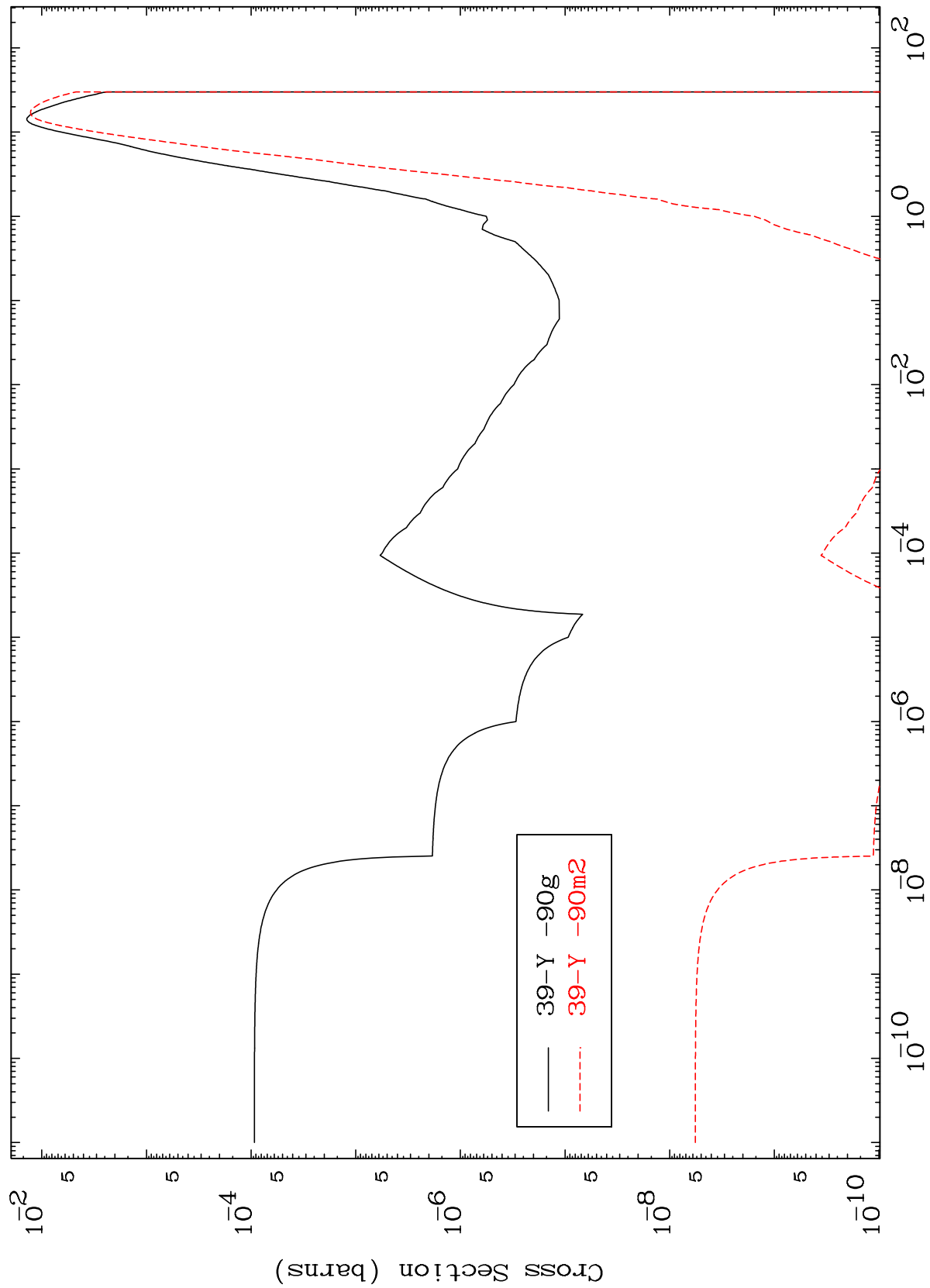




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41-Nb-93m

(n, α)
Radionuclide Production Cross Section



Incident Energy (MeV)

41-Nb-93m

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Radionuclide Production Cross Section
(n,2α)

