

Program EVALPLOT  
(Version 2021-1)

by

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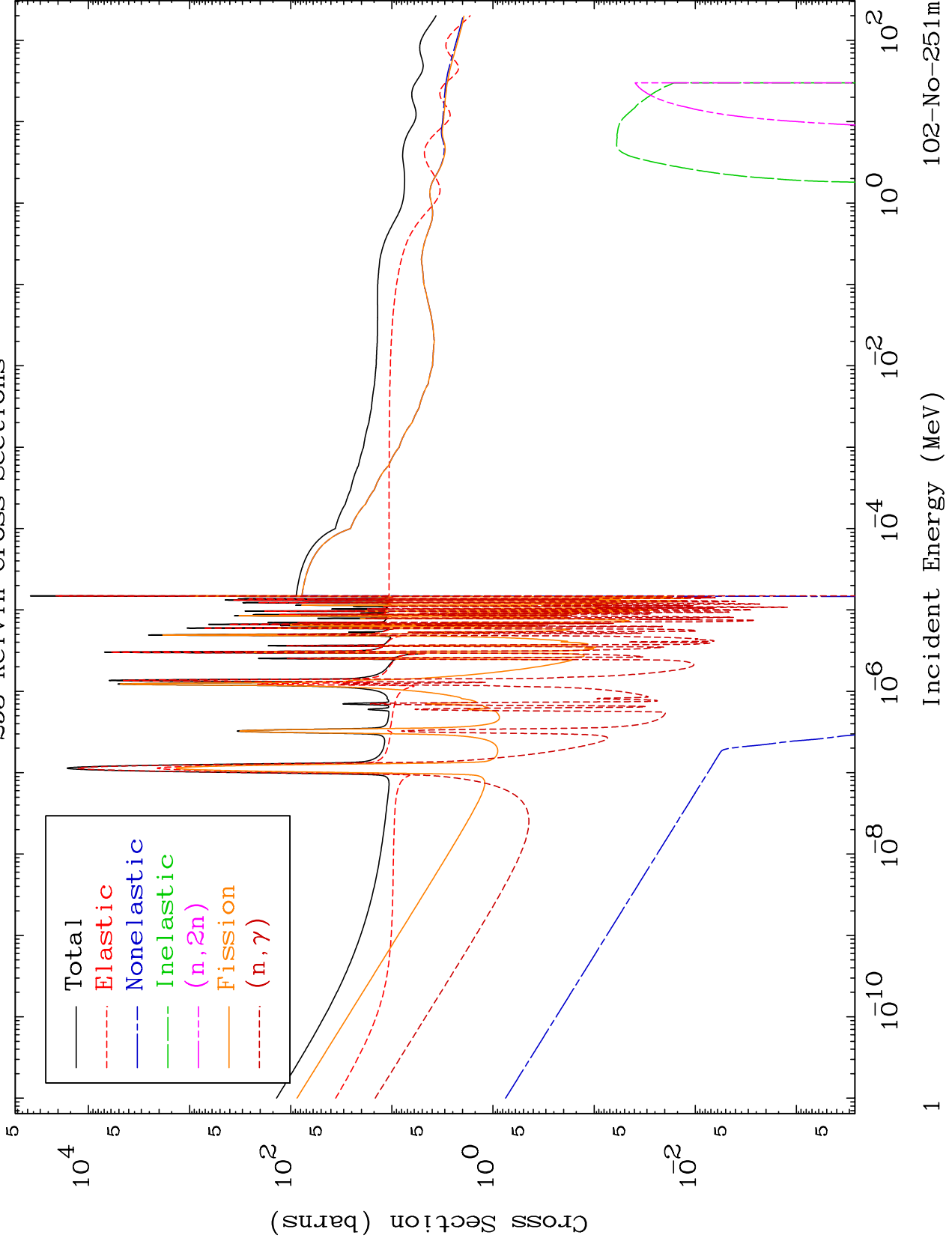
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9974

Neutron Major  
293 Kelvin Cross Sections

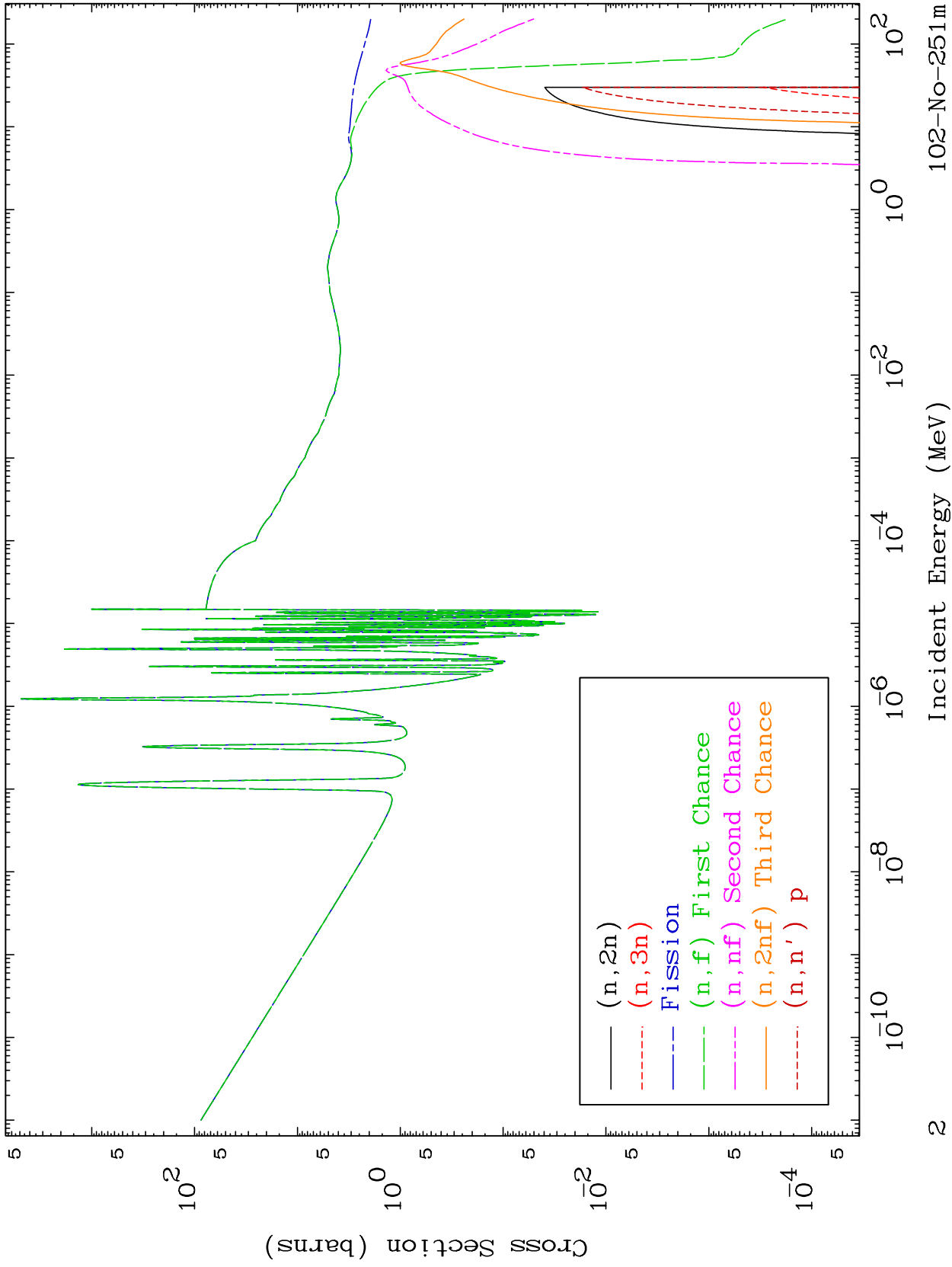
102-No-251m



MAT 9974

Neutron Absorption  
293 Kelvin Cross Sections

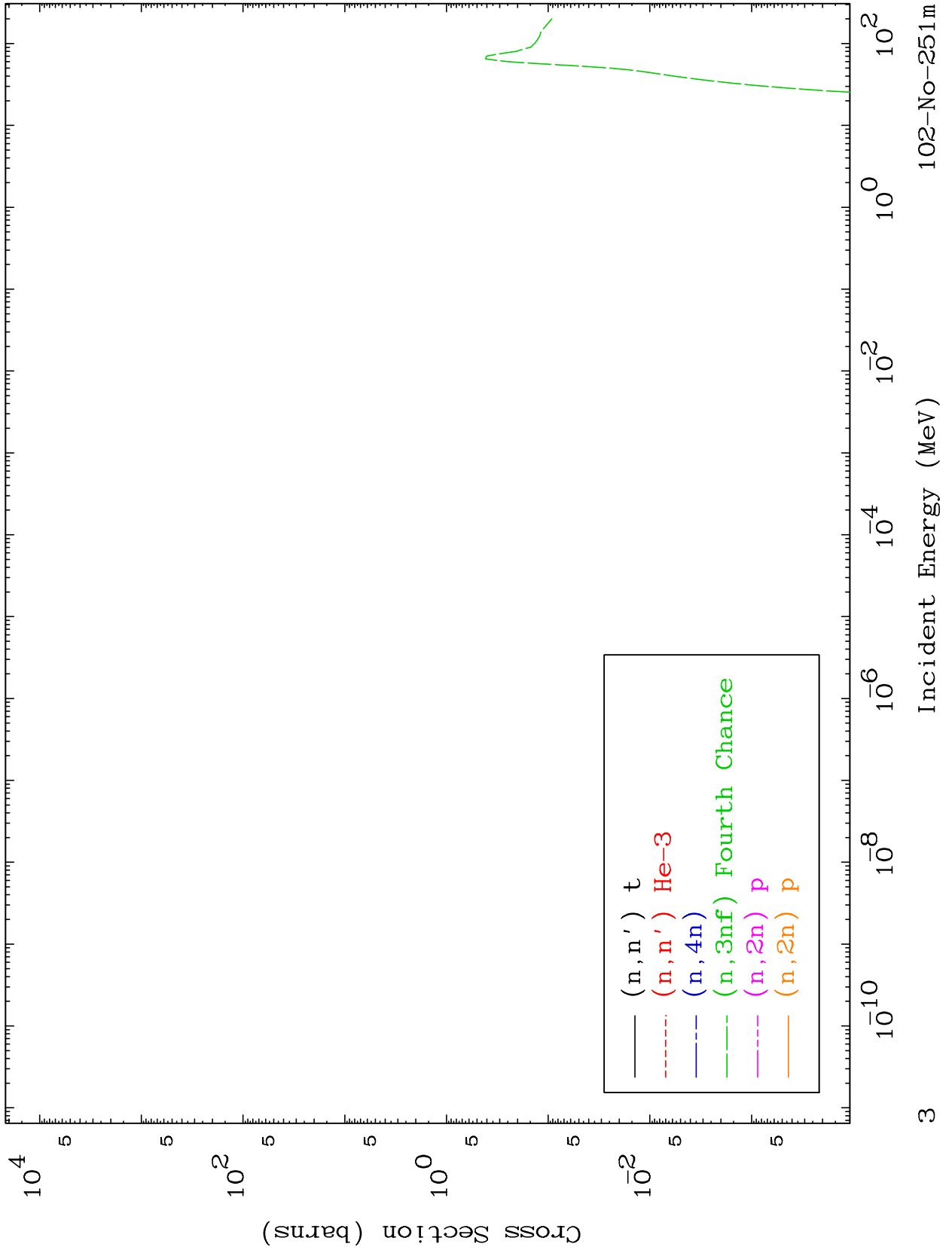
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Neutron Absorption  
293 Kelvin Cross Sections

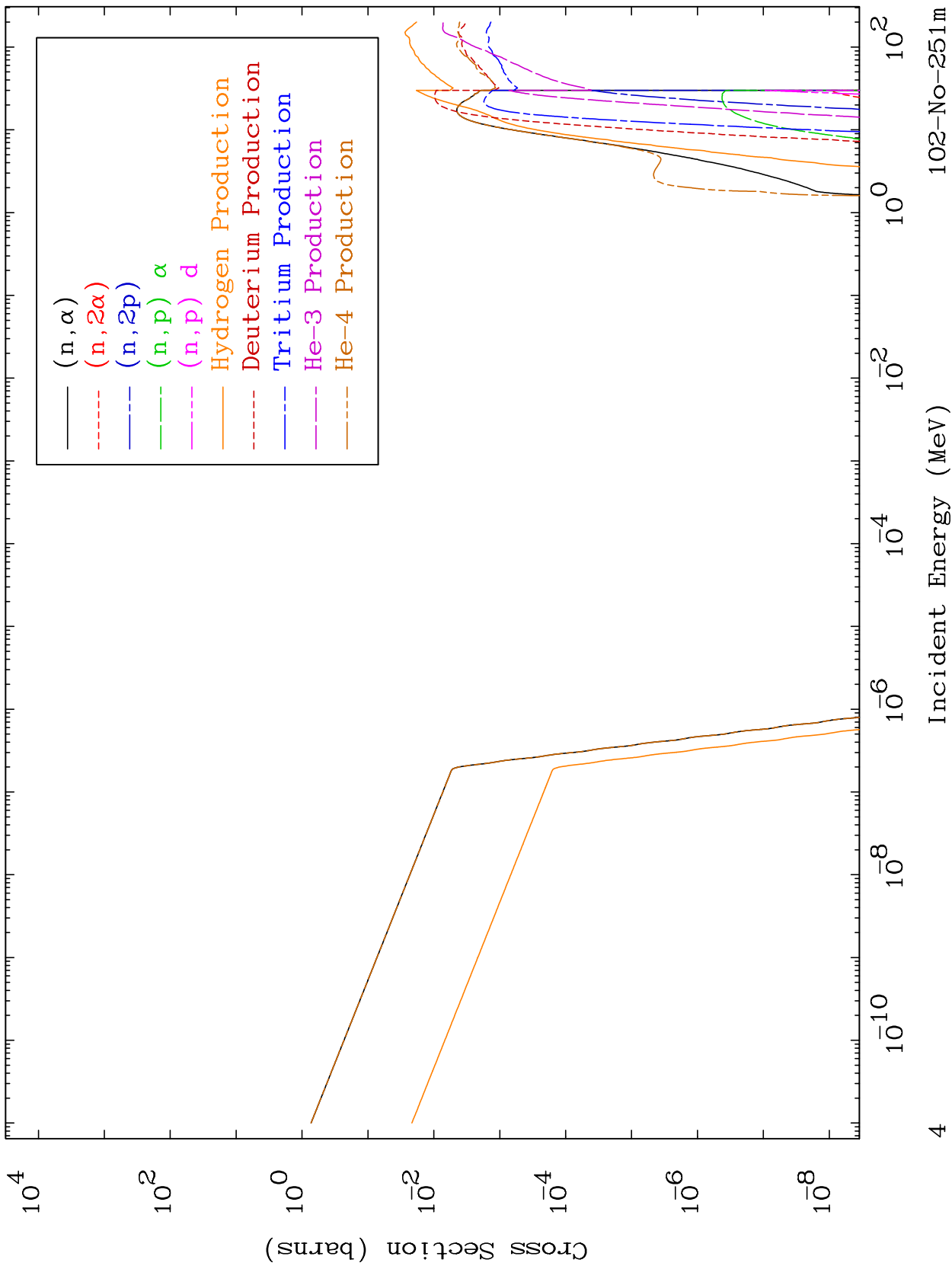
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Neutron Absorption  
293 Kelvin Cross Sections

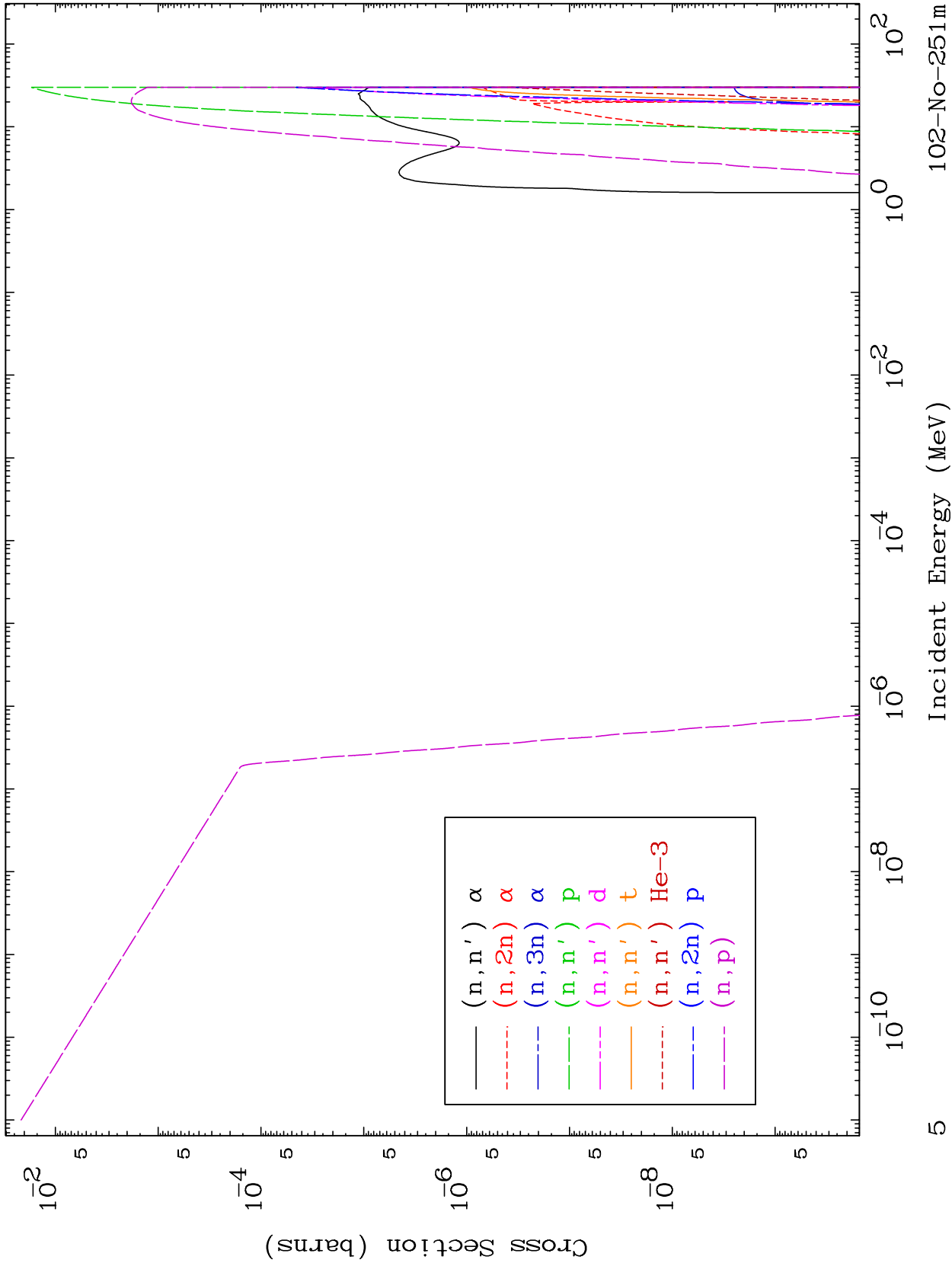
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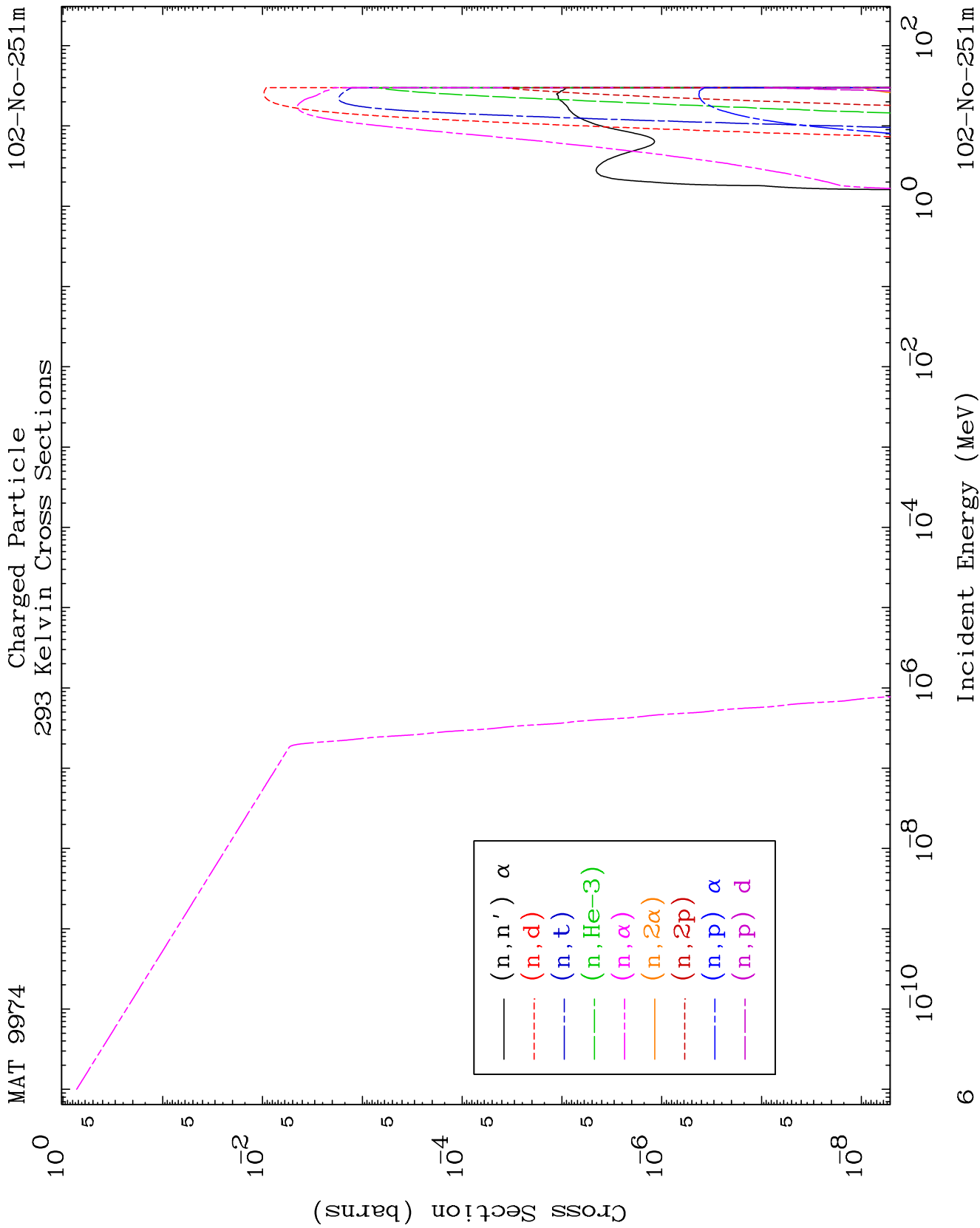


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Charged Particle  
293 Kelvin Cross Sections

102-No-251m

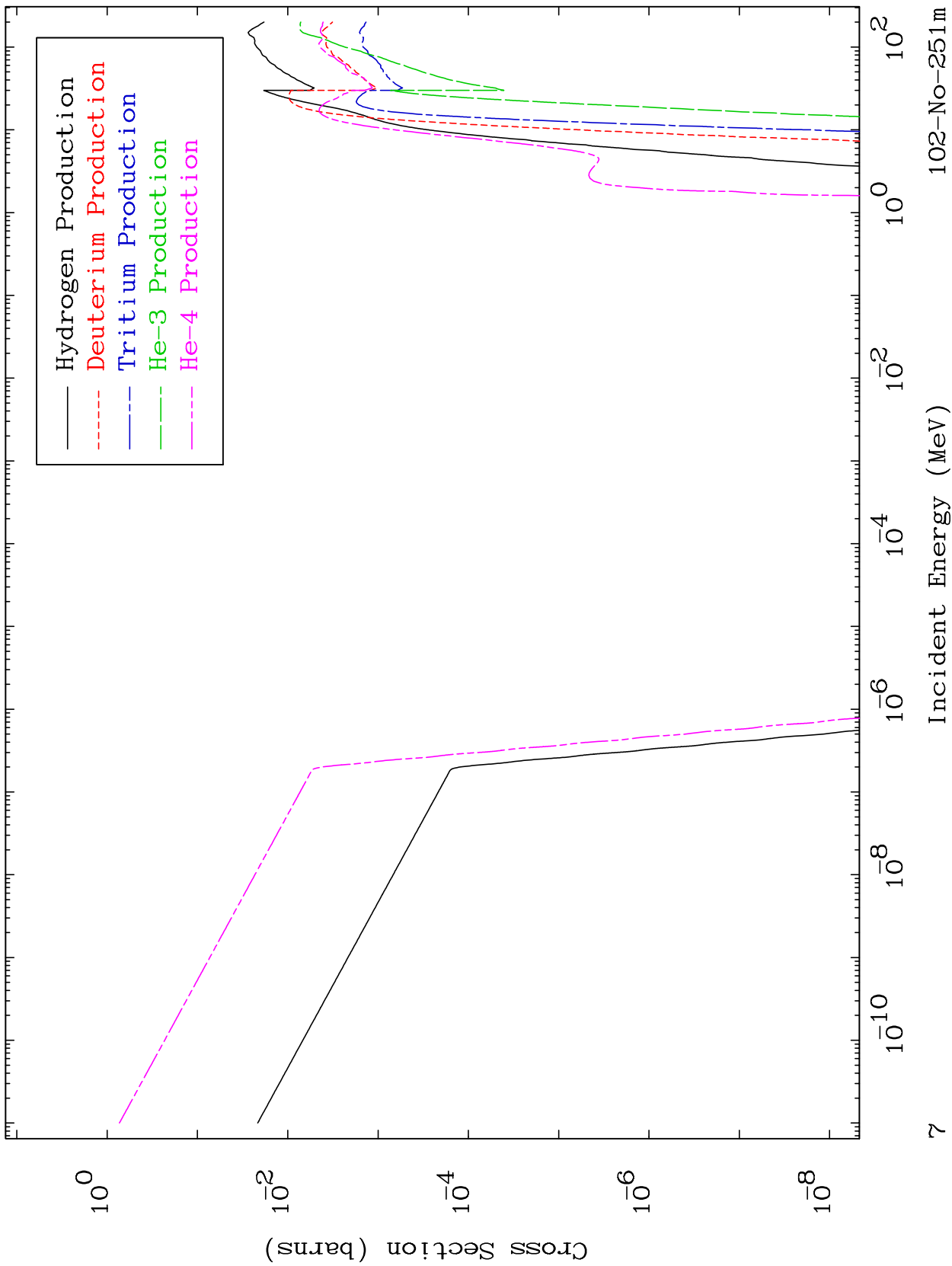




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Particle Production  
293 Kelvin Cross Sections

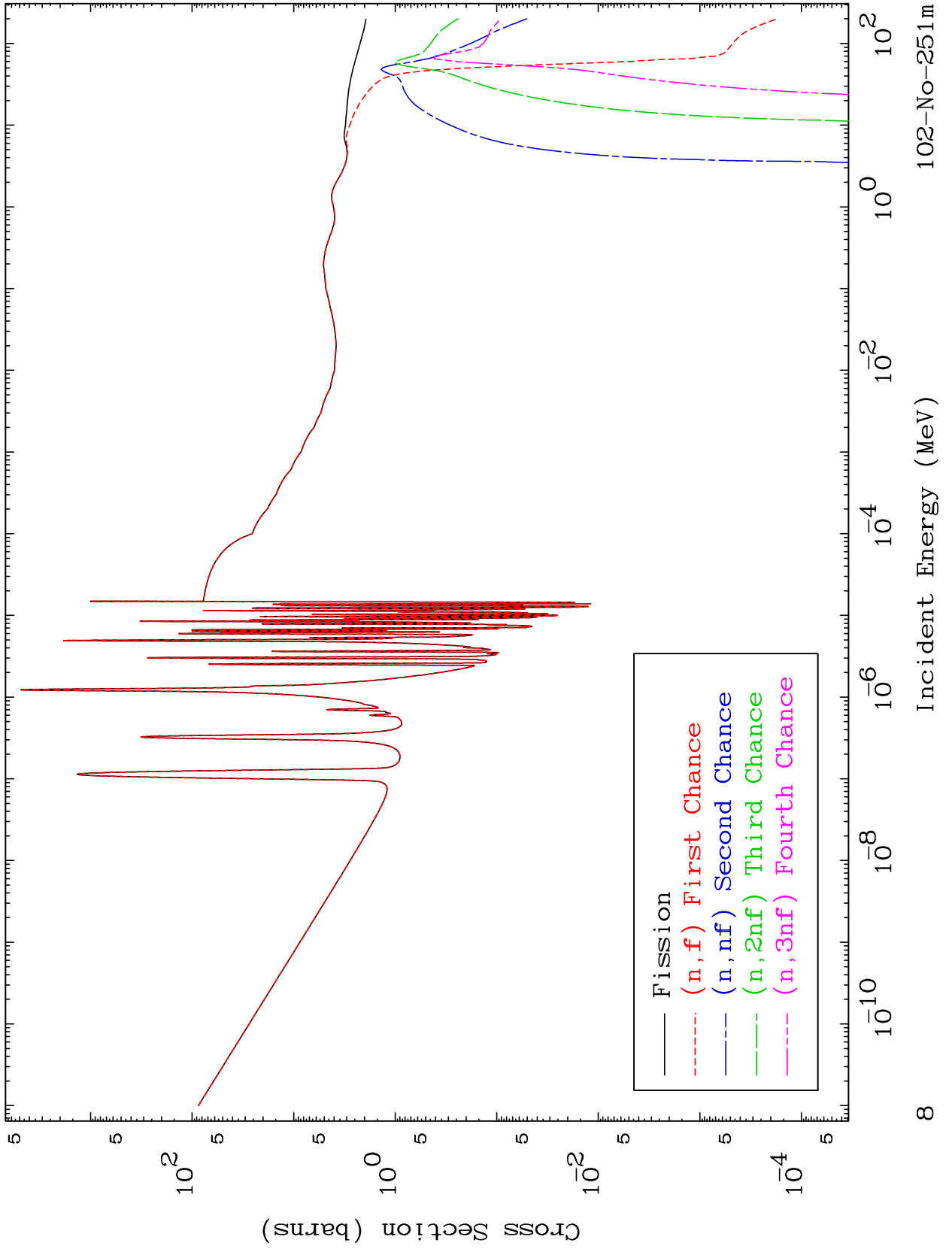
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Fission  
293 Kelvin Cross Sections

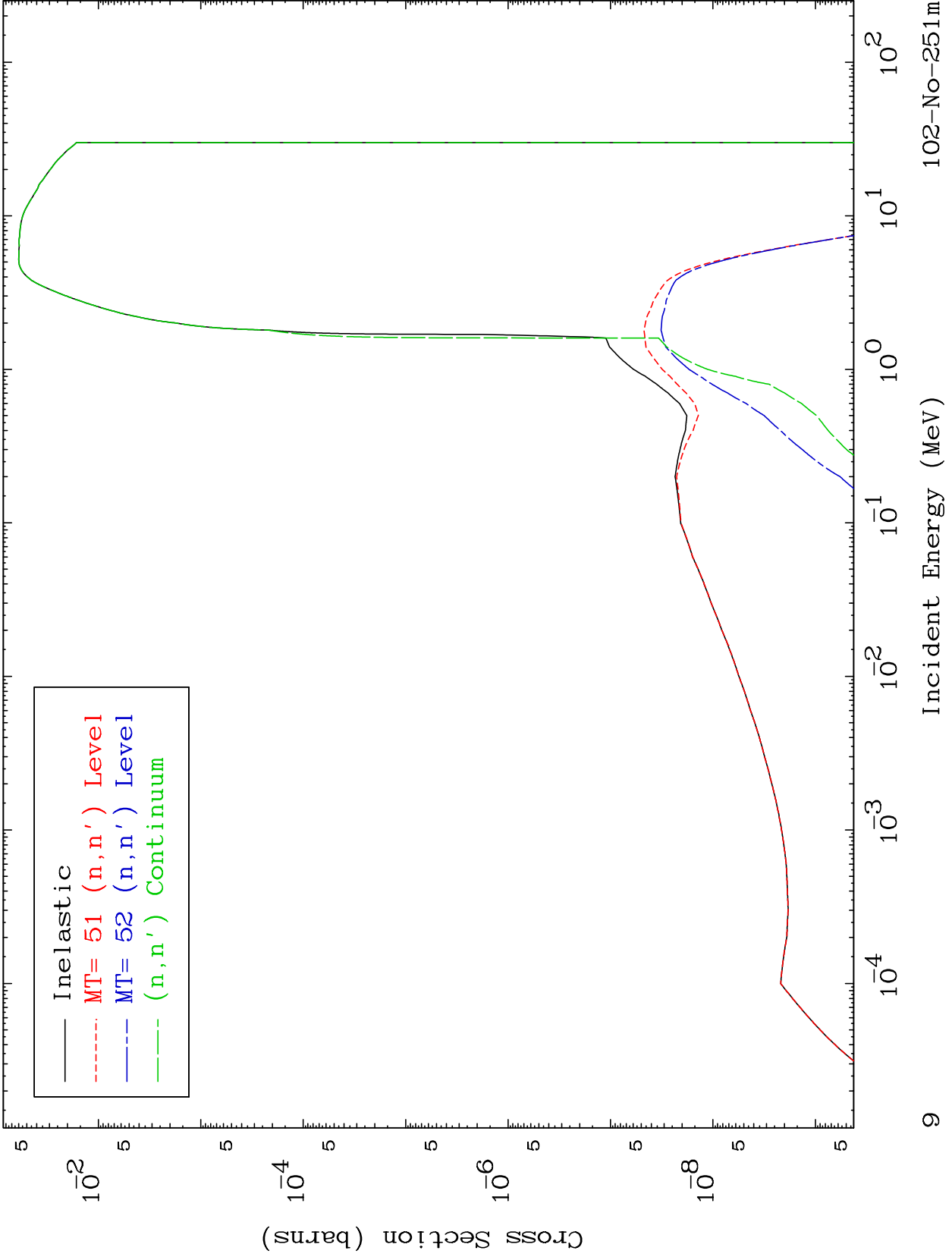
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(n,n') Levels  
293 Kelvin Cross Sections

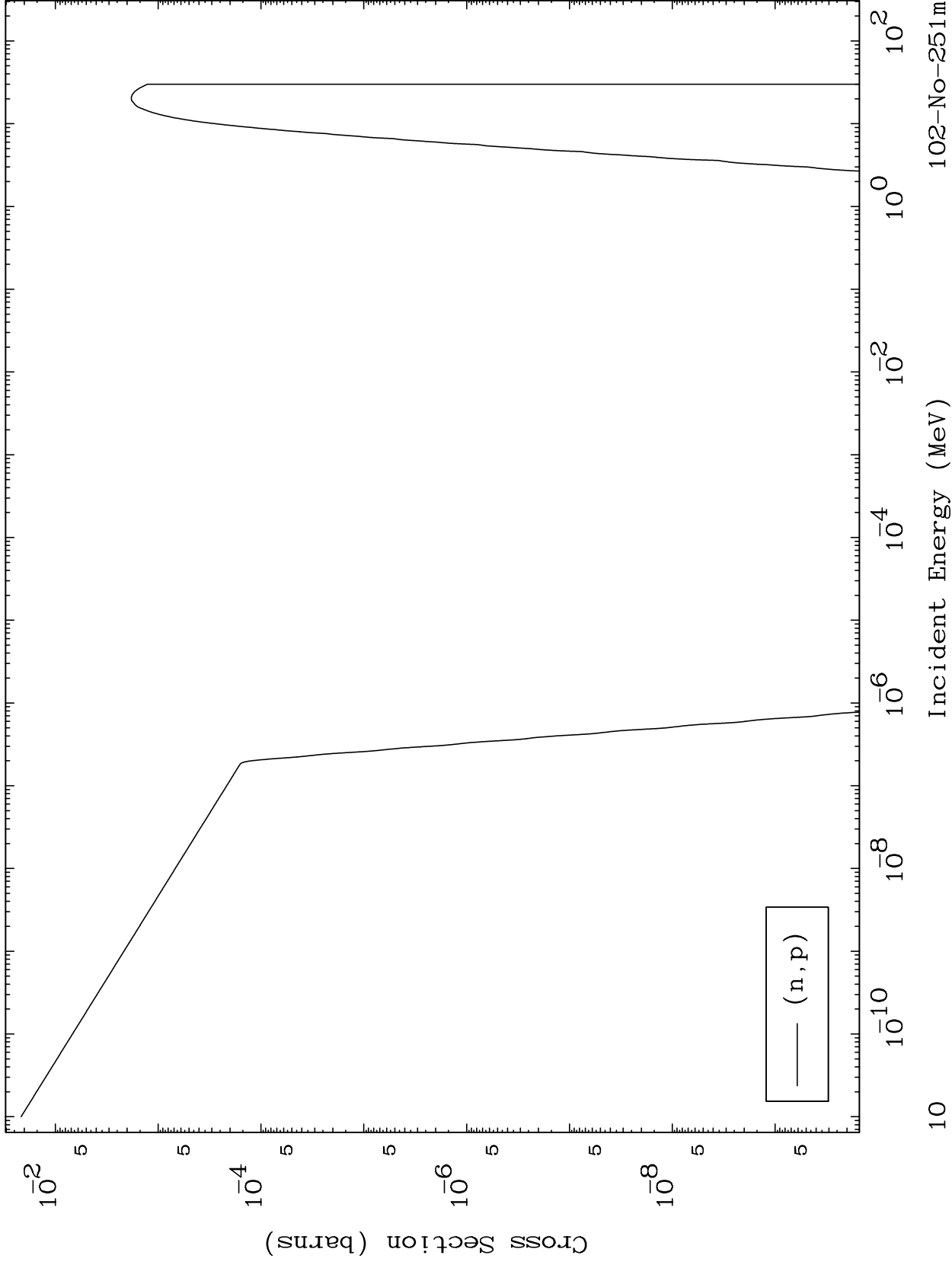
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(n,p) Levels  
293 Kelvin Cross Sections

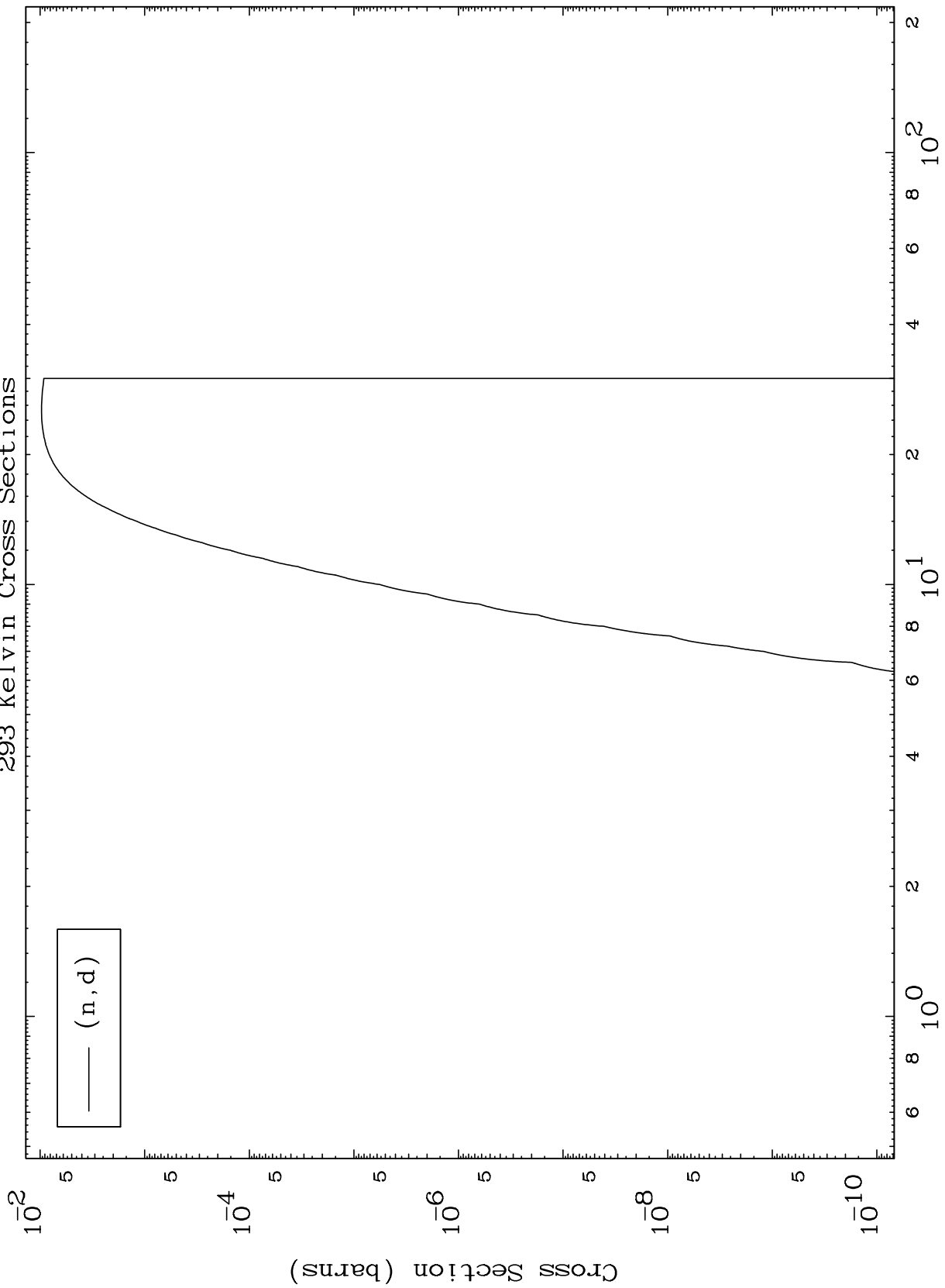
102-No-251m



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(n,d) Levels  
293 Kelvin Cross Sections

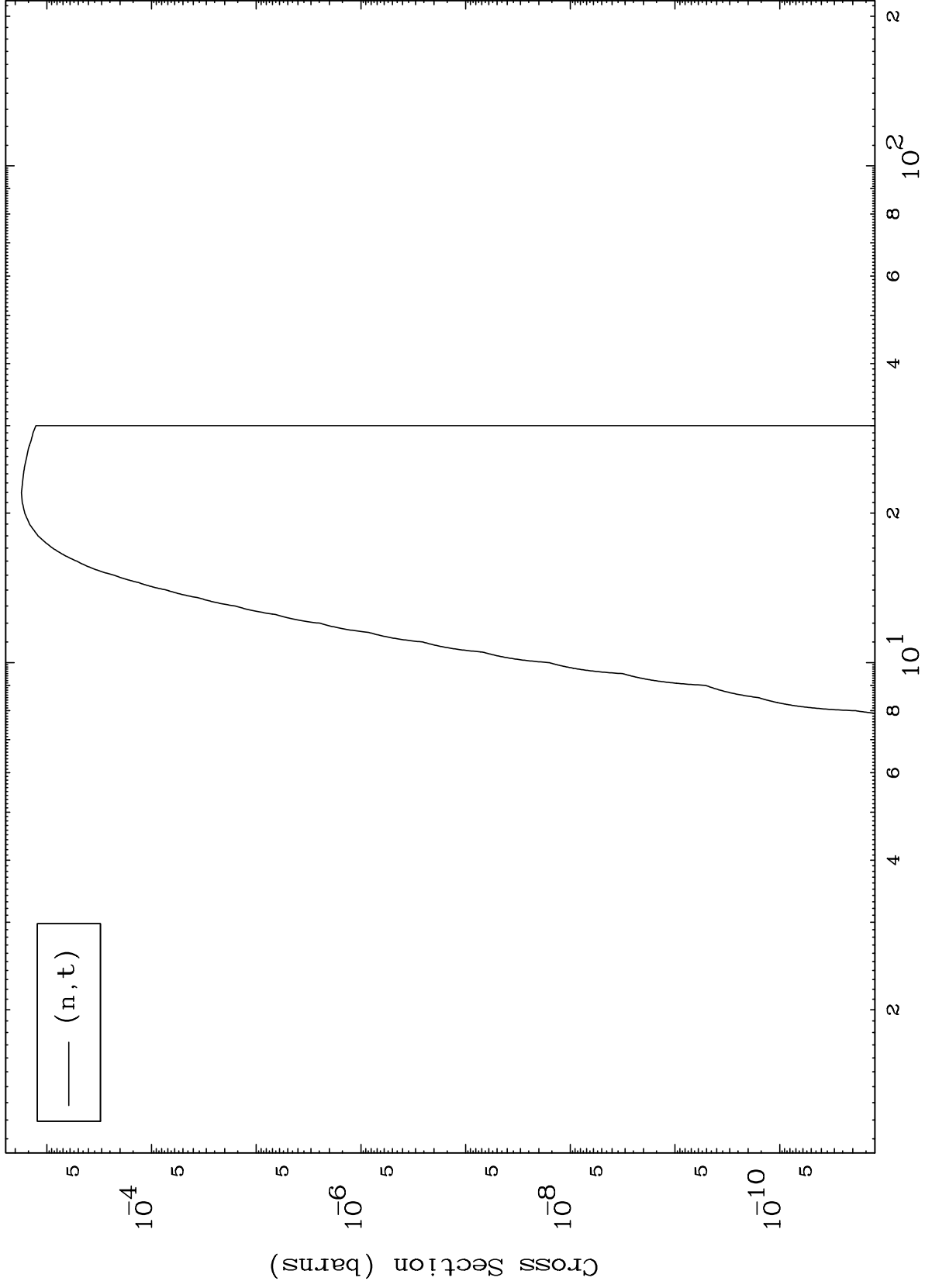
102-No-251m



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(n,t) Levels  
293 Kelvin Cross Sections

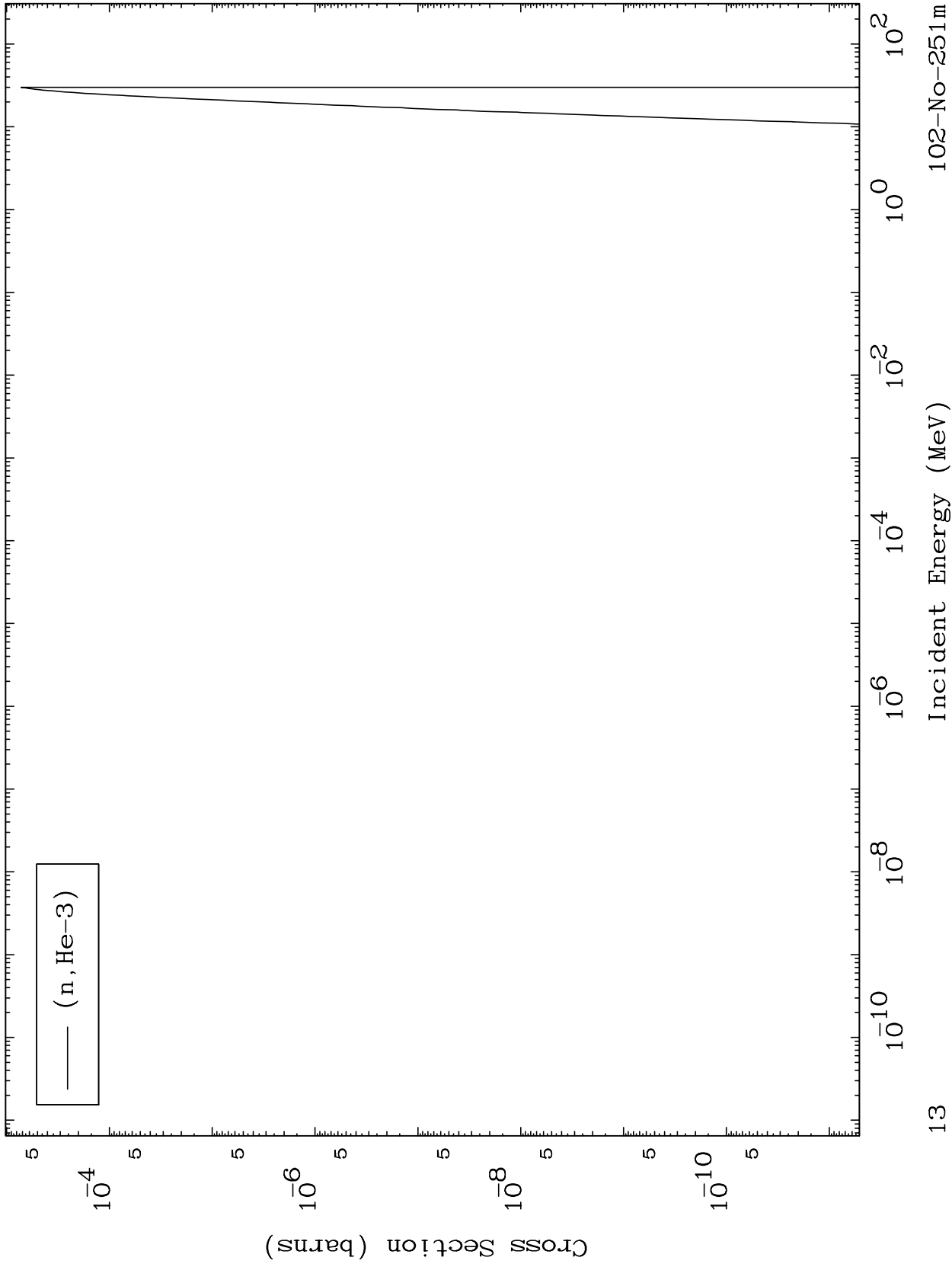
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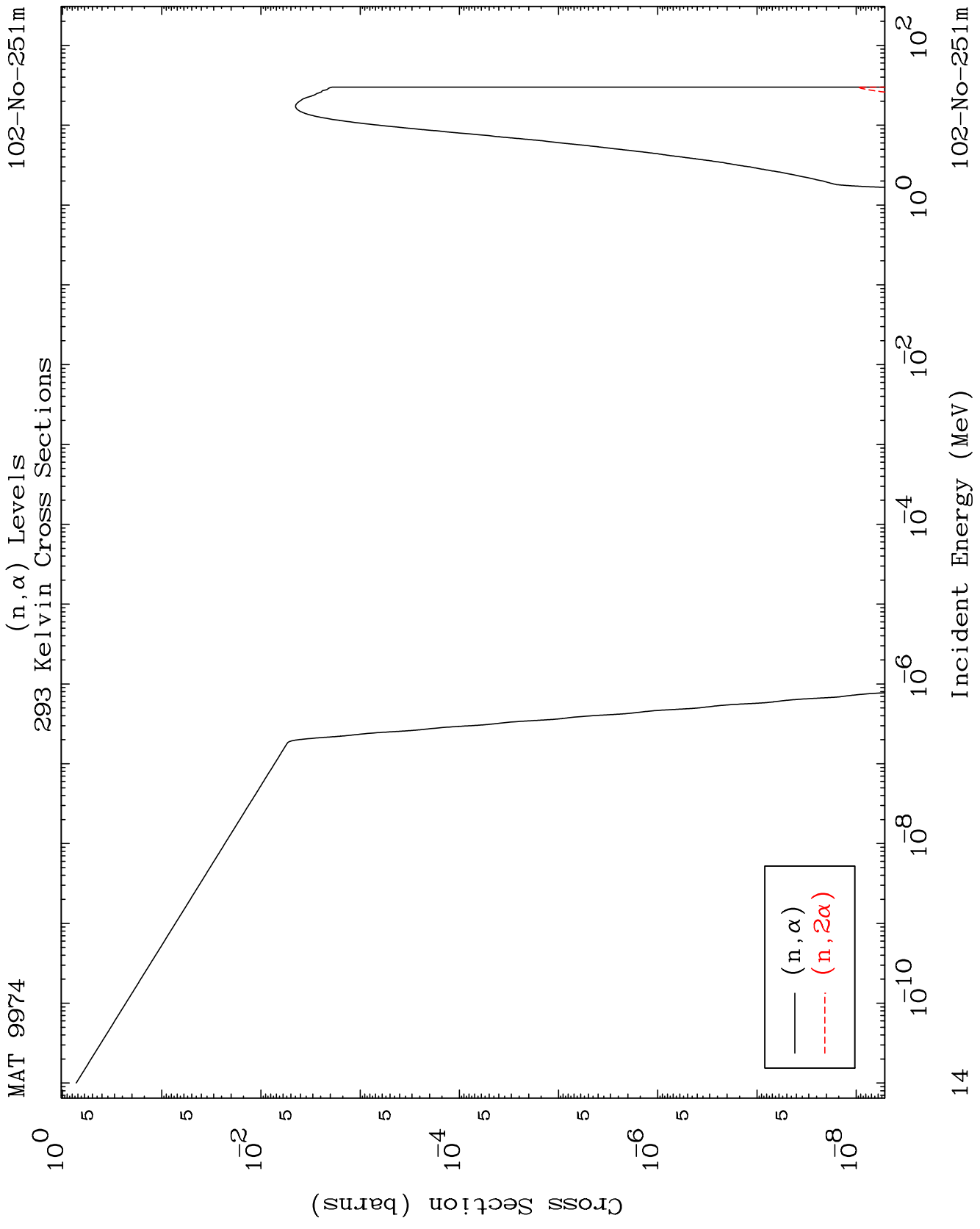


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(n,He3) Levels  
293 Kelvin Cross Sections

102-No-251m

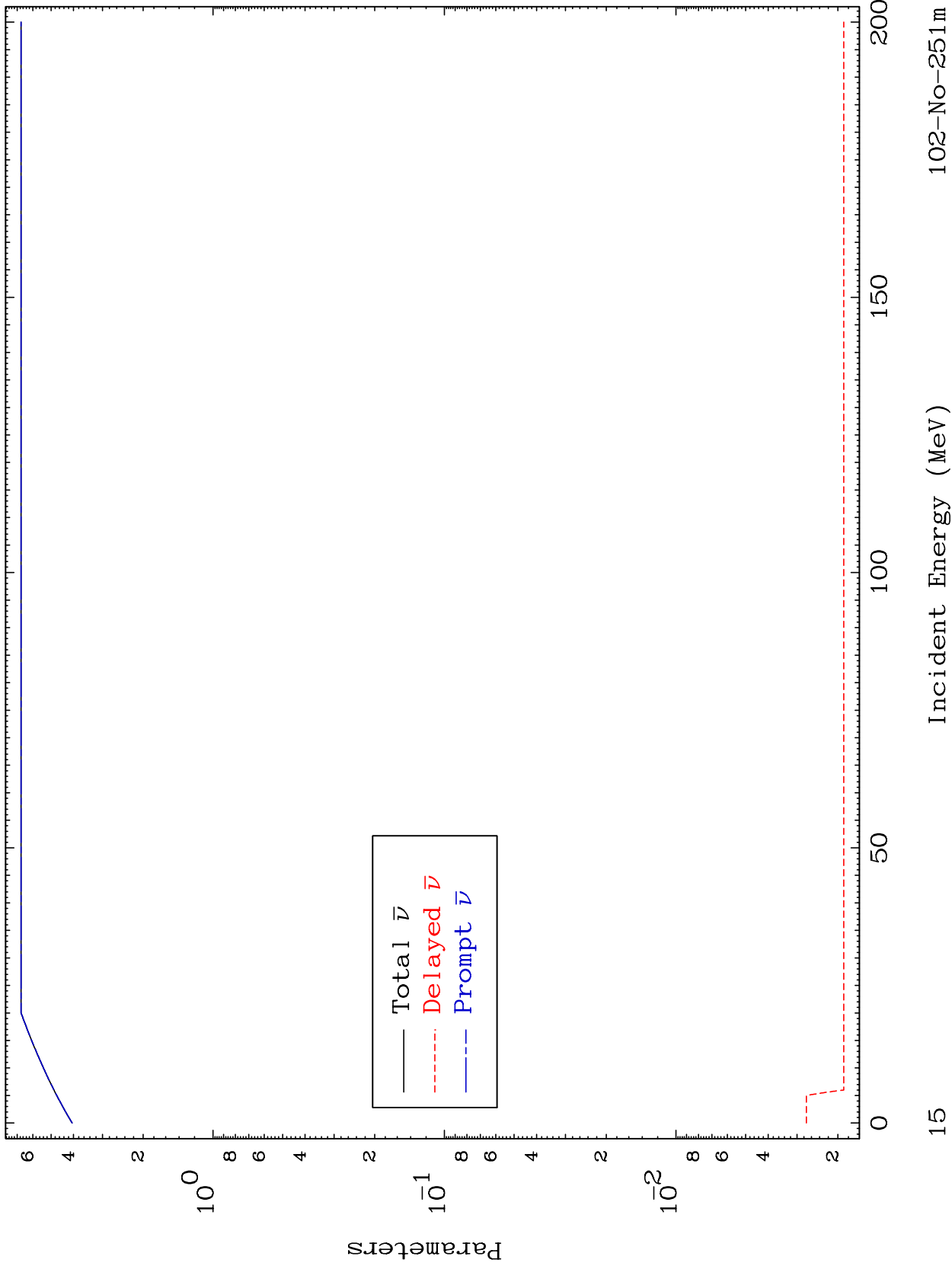




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Energy Release  
Parameters

102-No-251m



15

Incident Energy (MeV)

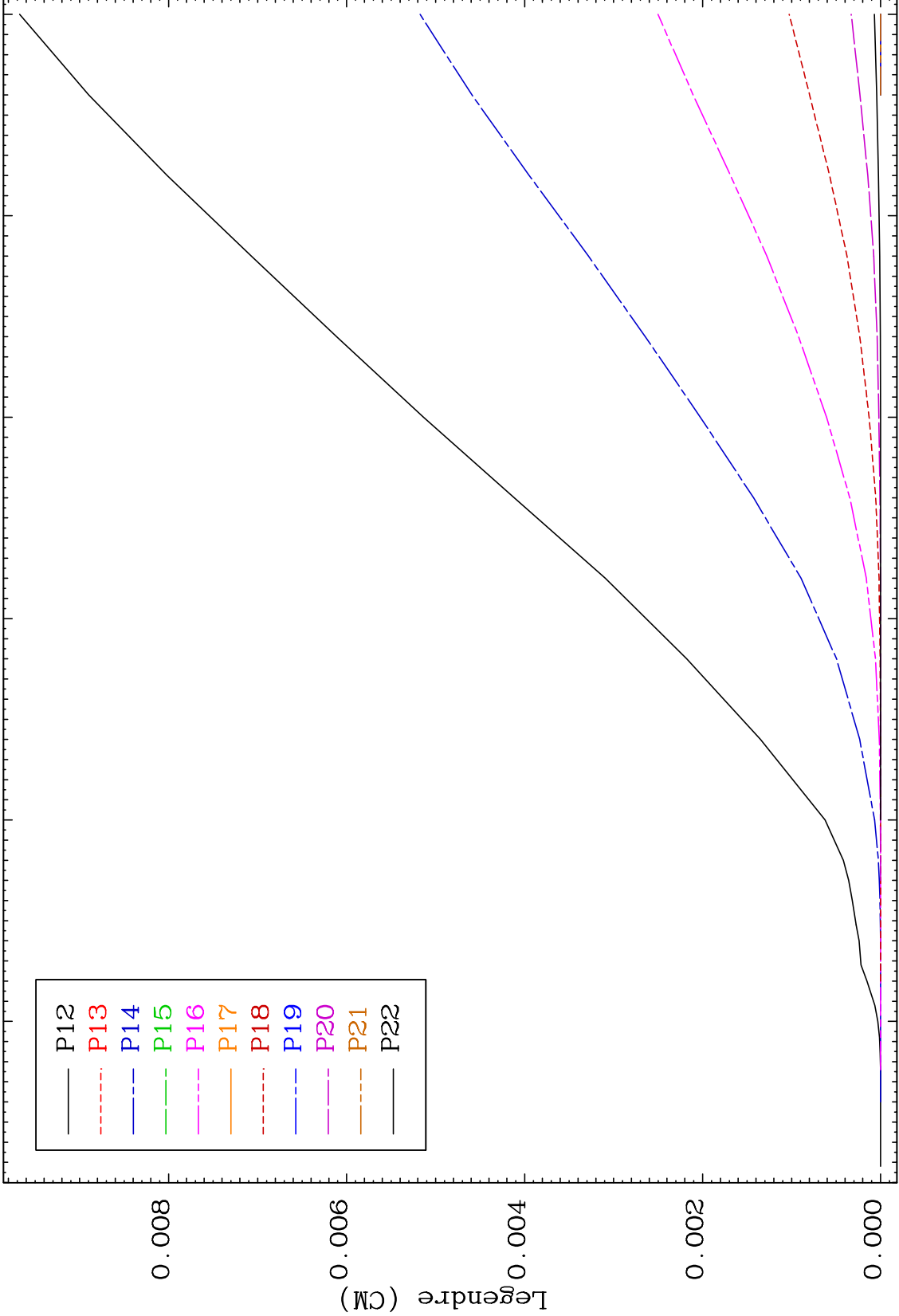
102-No-251m



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Elastic Legendre Coefficients

102-No-251m



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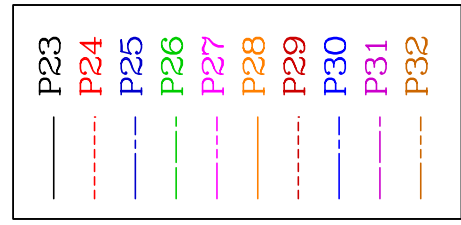
102-No-251m

Incident Energy (MeV)

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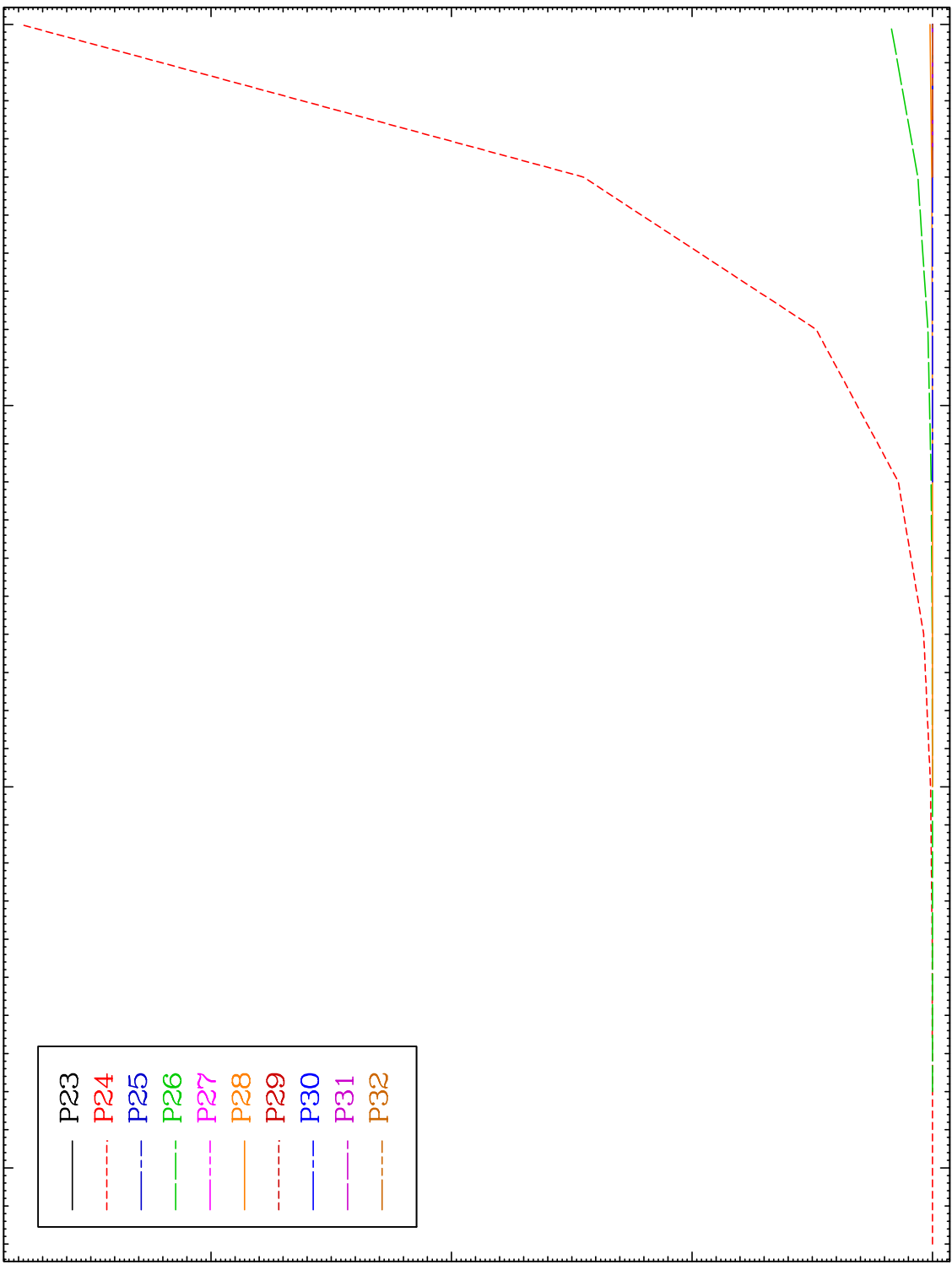
Elastic Legendre Coefficients

102-No-251m



$\times 10^{-6}$

Legendre (CM)



30

25

20

15

Incident Energy (MeV)

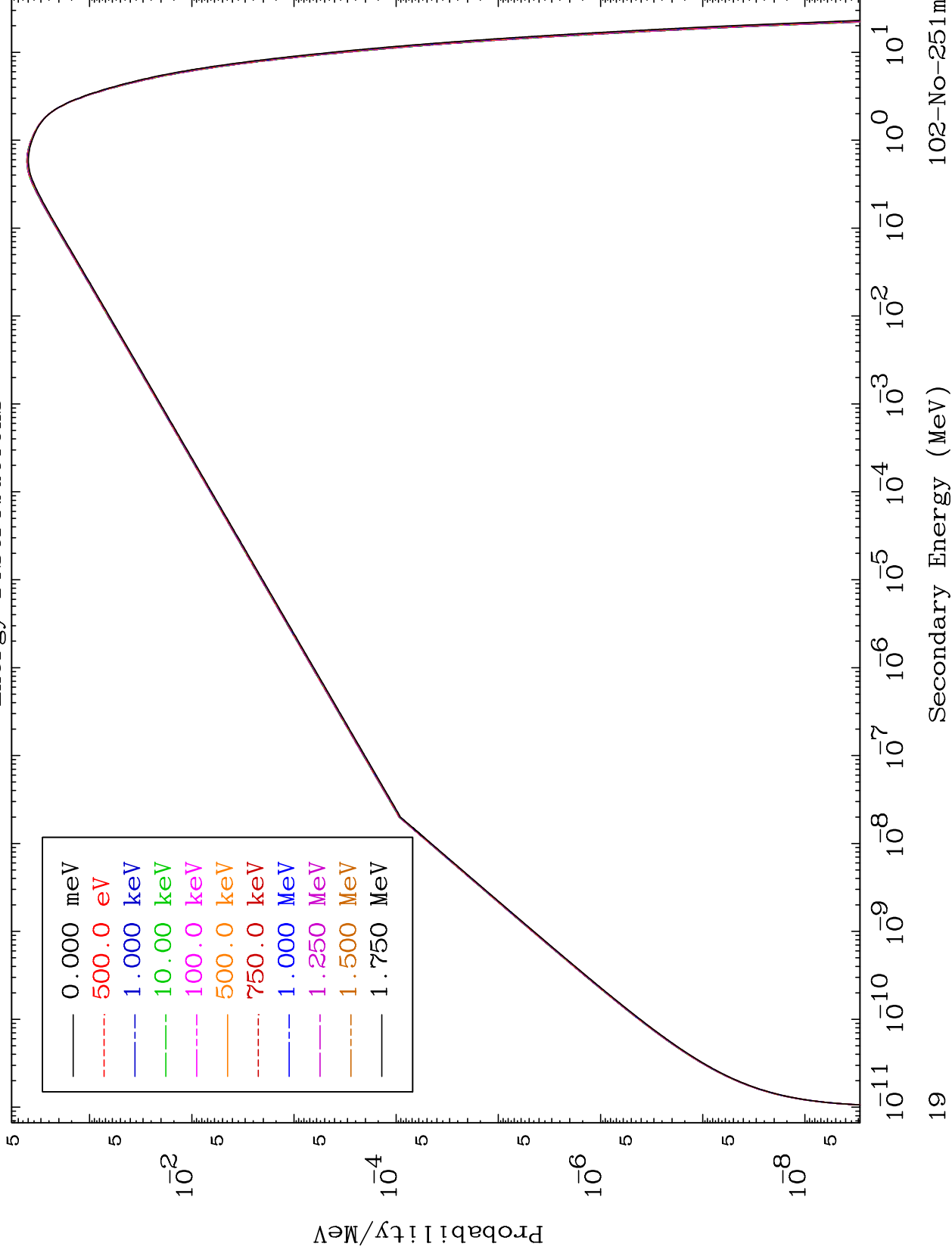
102-No-251m

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### Fission Energy Distributions

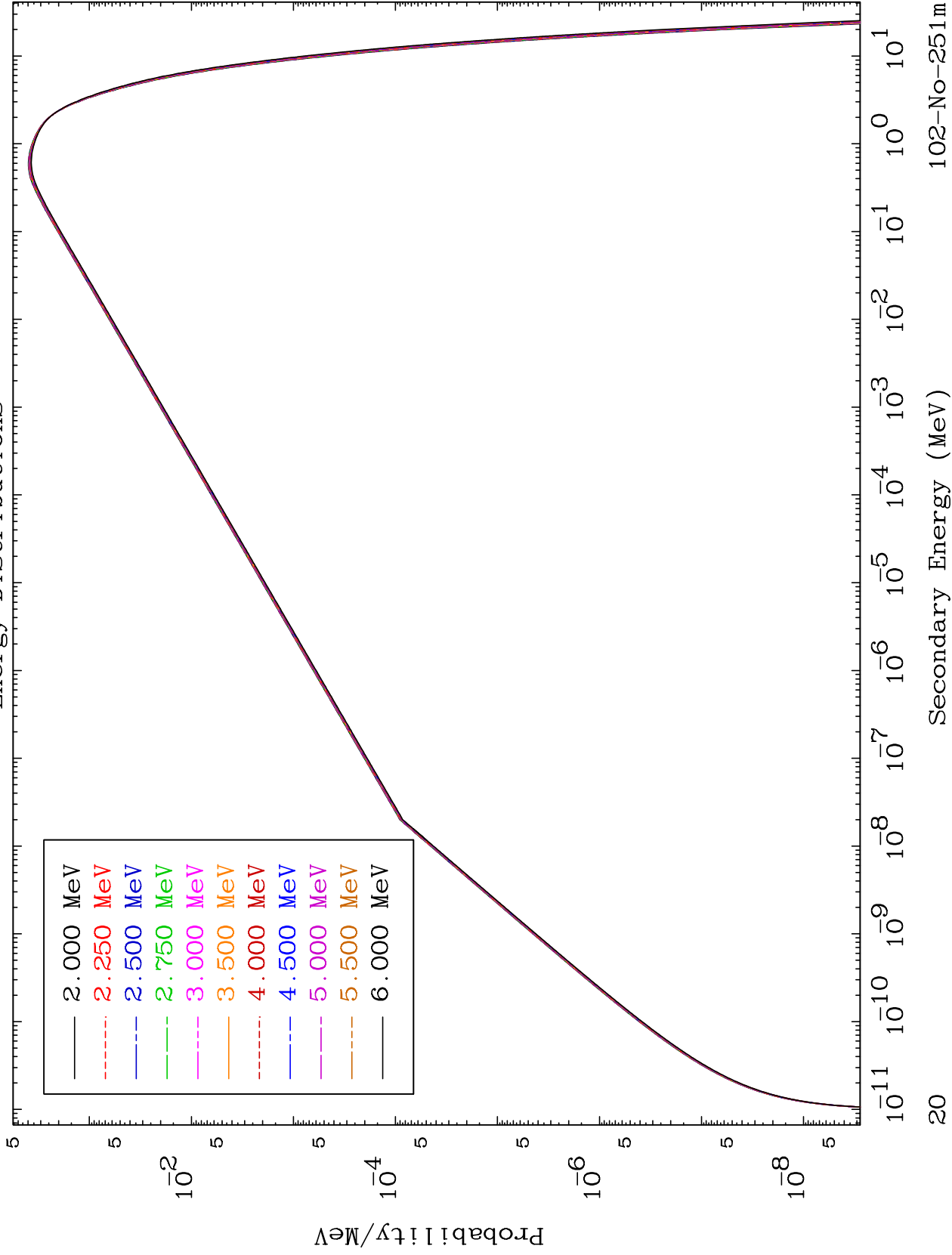
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### Fission Energy Distributions

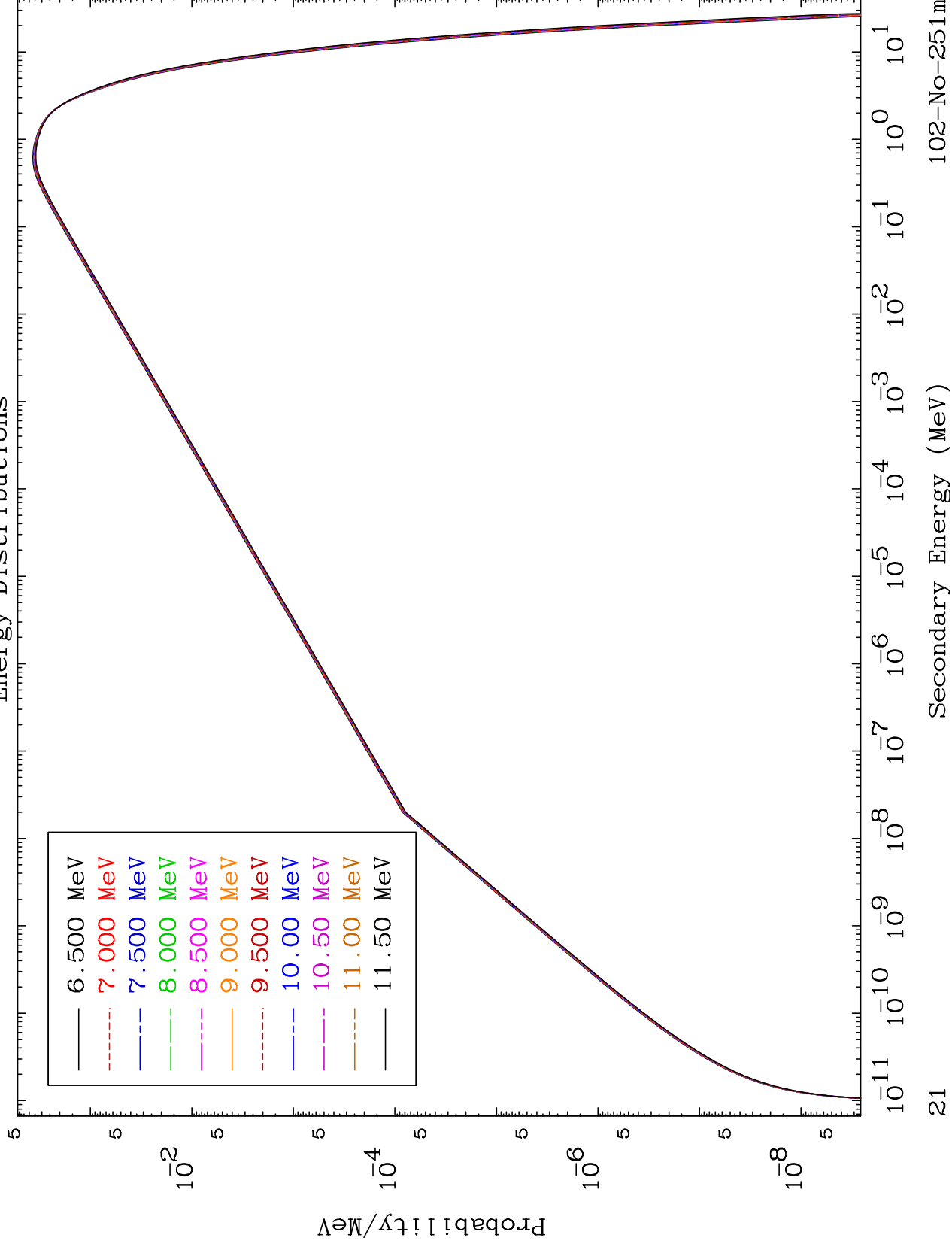
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### Fission Energy Distributions

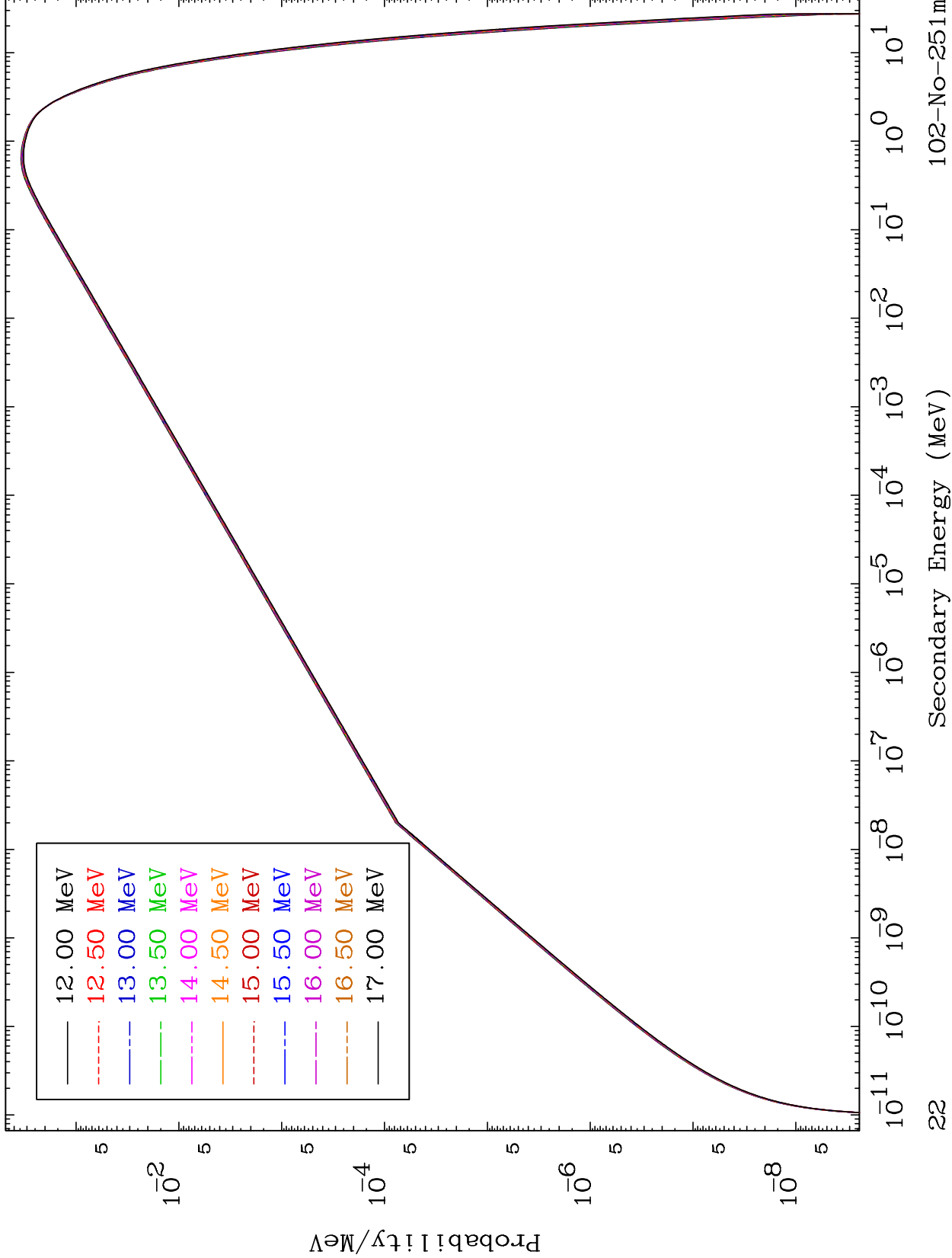
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### Fission Energy Distributions

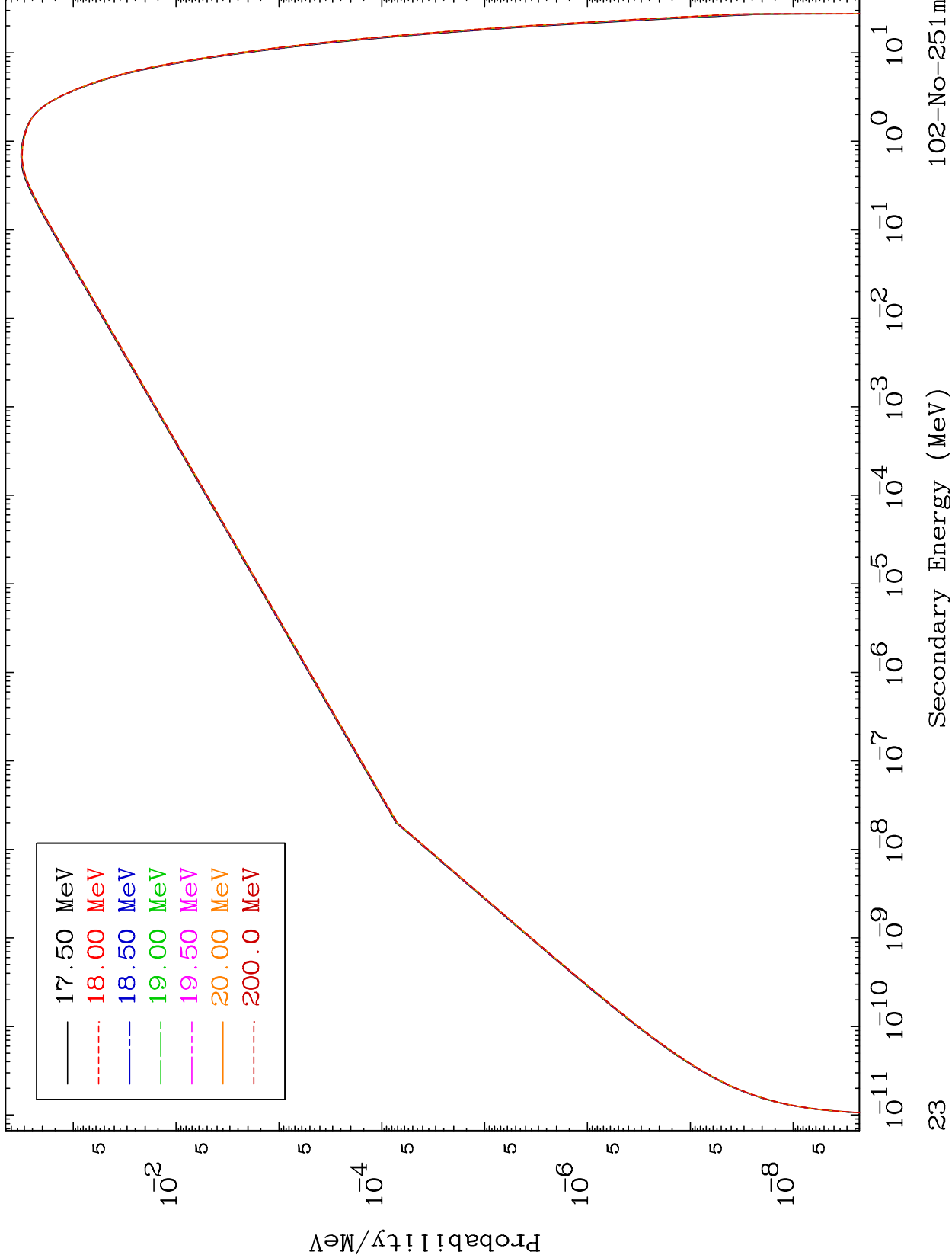
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### Fission Energy Distributions

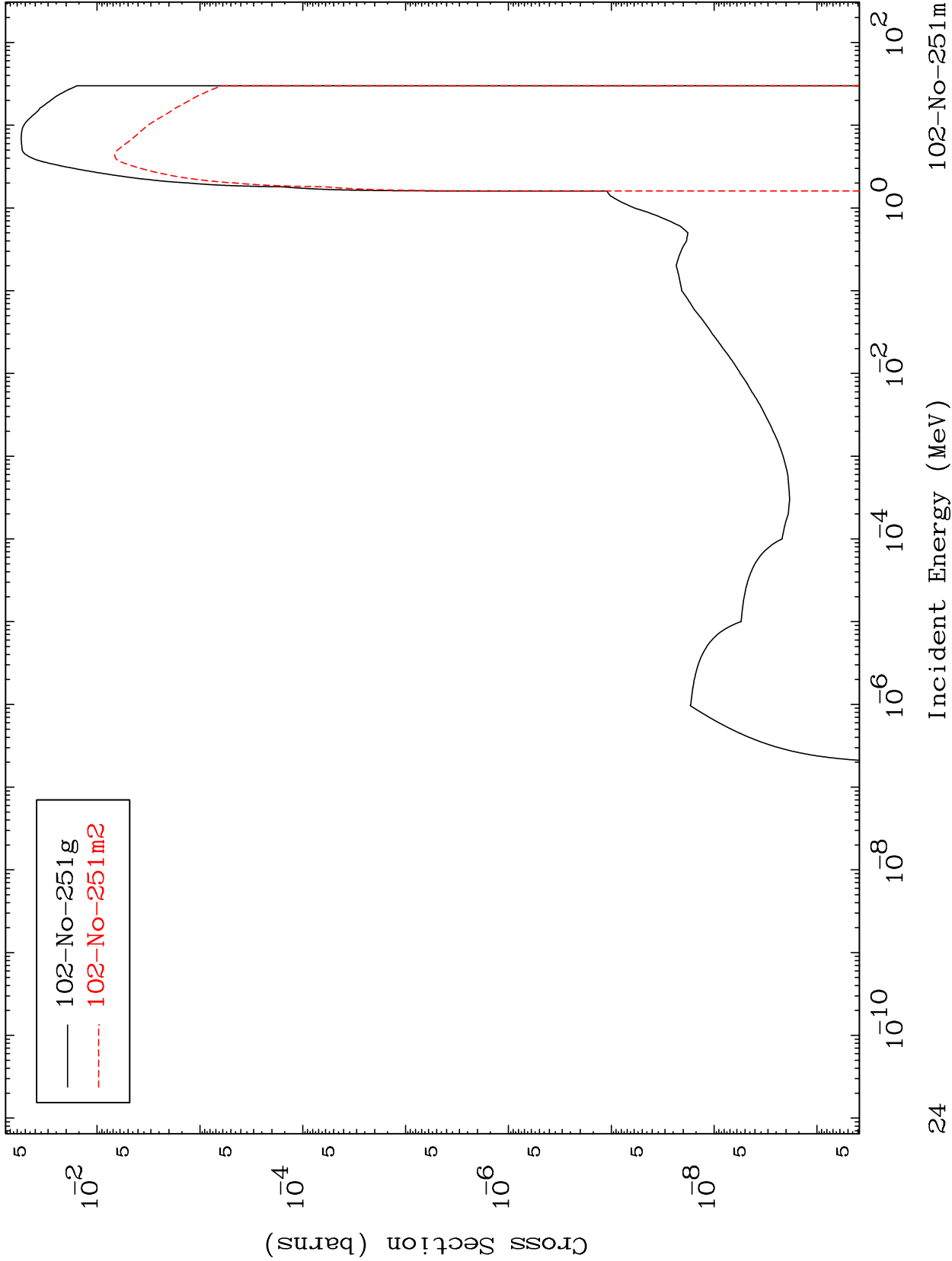
102-No-251m



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Inelastic  
Radionuclide Production Cross Section

102-No-251m

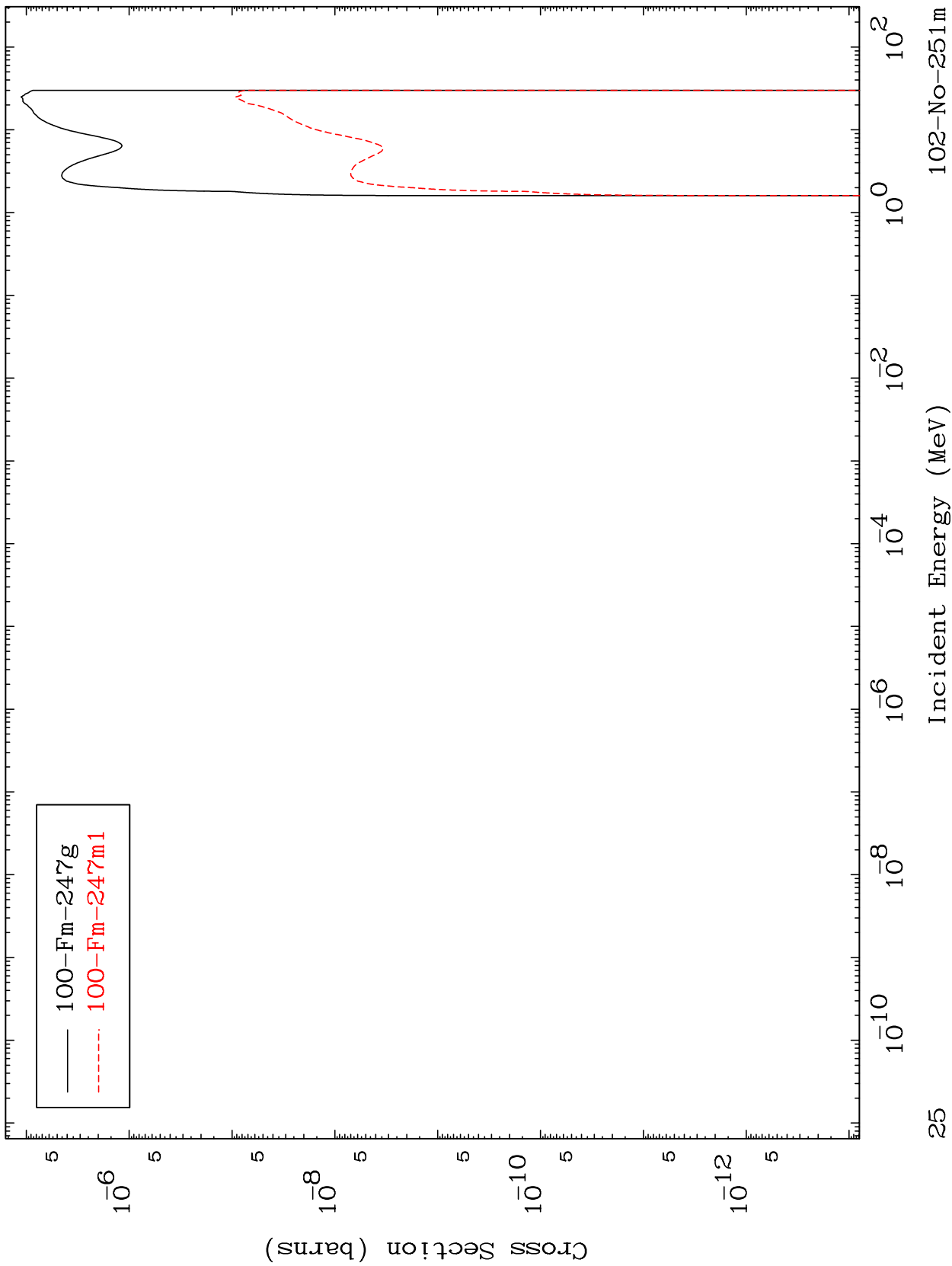


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$(n, n') \alpha$

102-No-251m

Radionuclide Production Cross Section

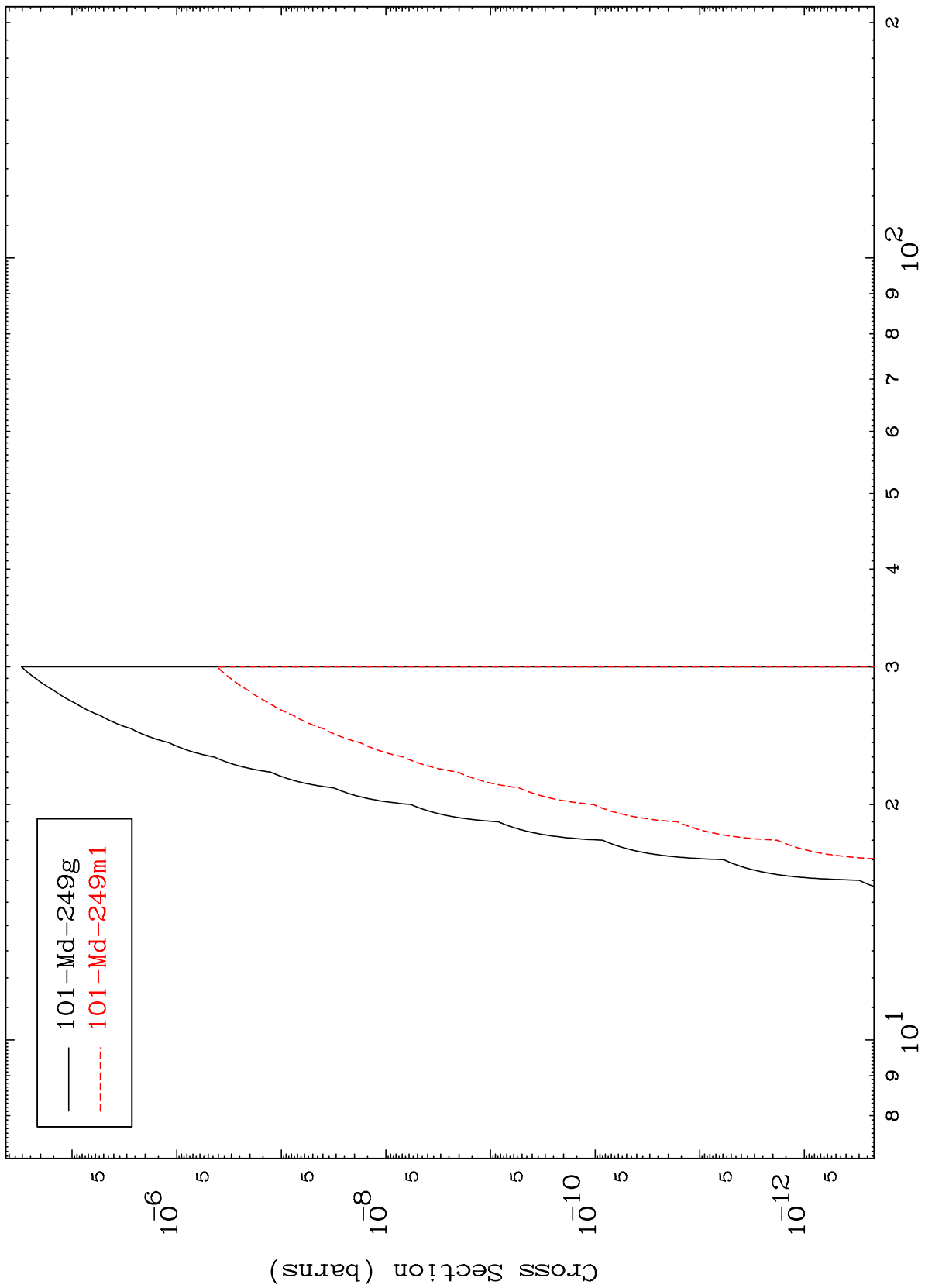


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(n,n') d

102-No-251m

Radionuclide Production Cross Section



26

Incident Energy (MeV)

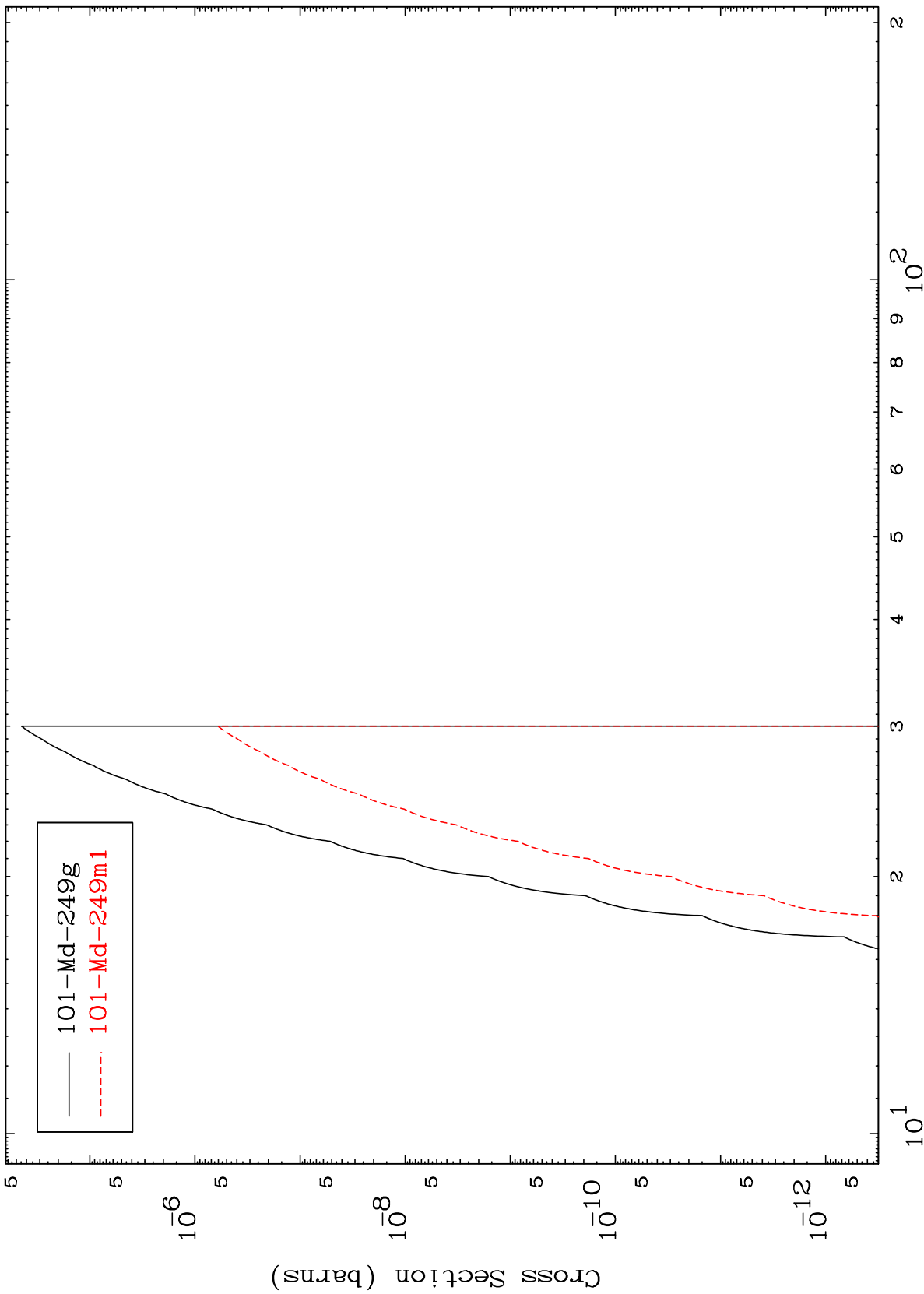
102-No-251m

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(n,2n) p

102-No-251m

Radionuclide Production Cross Section



101-Md-249g  
101-Md-249m1

Incident Energy (MeV)

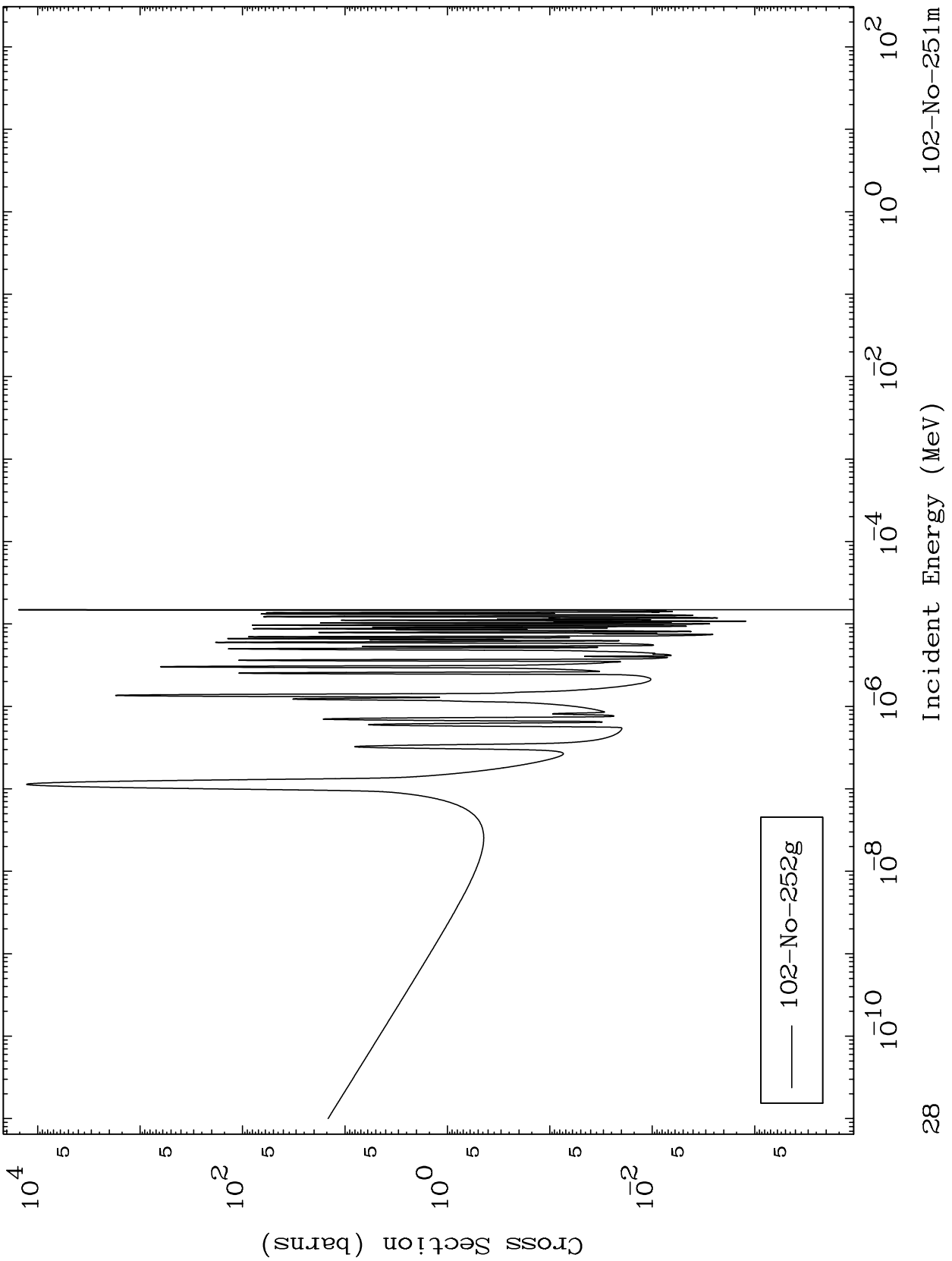
102-No-251m

27

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102-No-251m

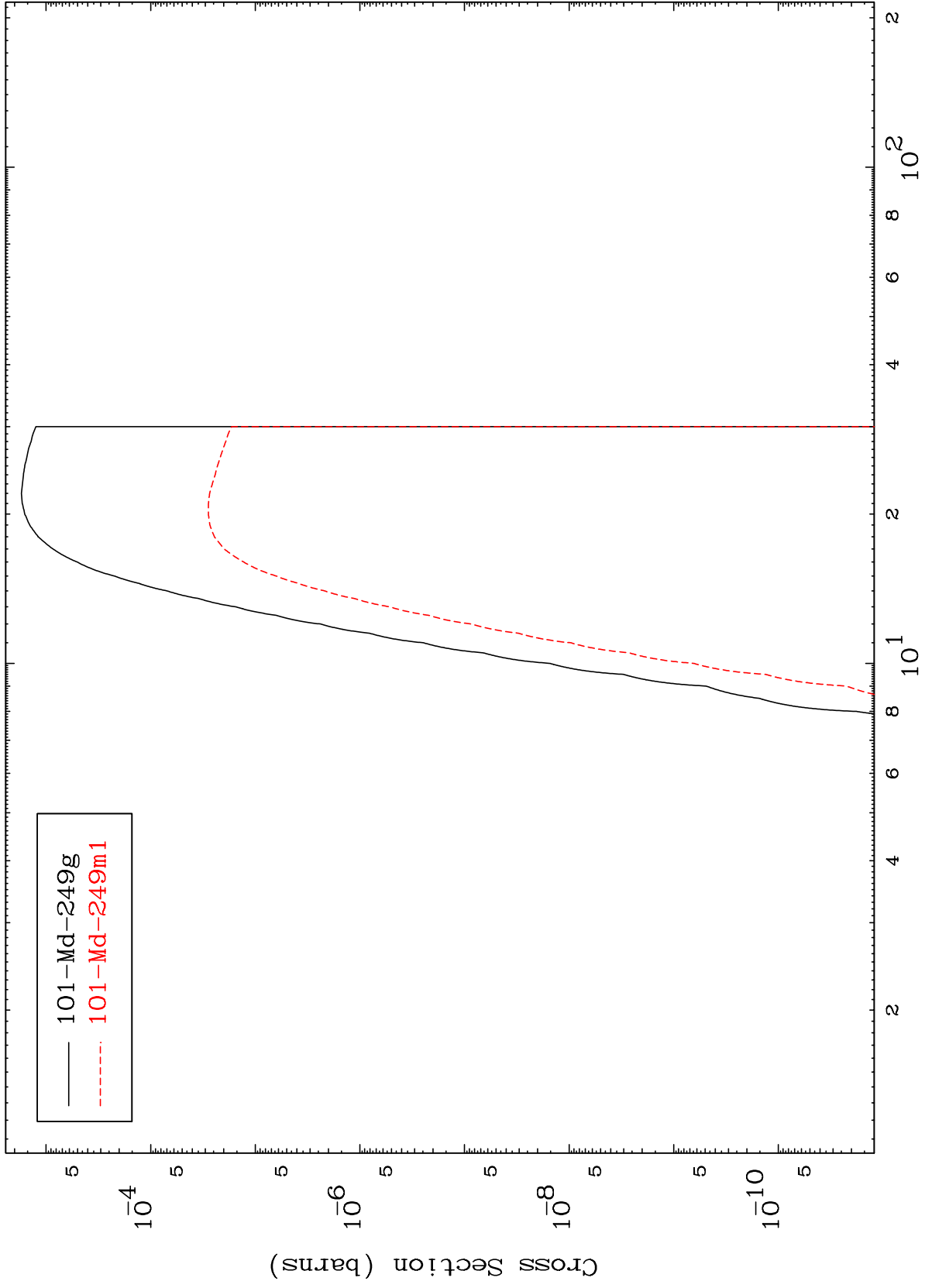
Radionuclide Production Cross Section  
(n,  $\gamma$ )



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102-No-251m

(n, t)  
Radionuclide Production Cross Section



MAT 9974

(n,p)  $\alpha$

102-No-251m

Radionuclide Production Cross Section

