

Program EVALPLOT
(Version 2021-1)

by

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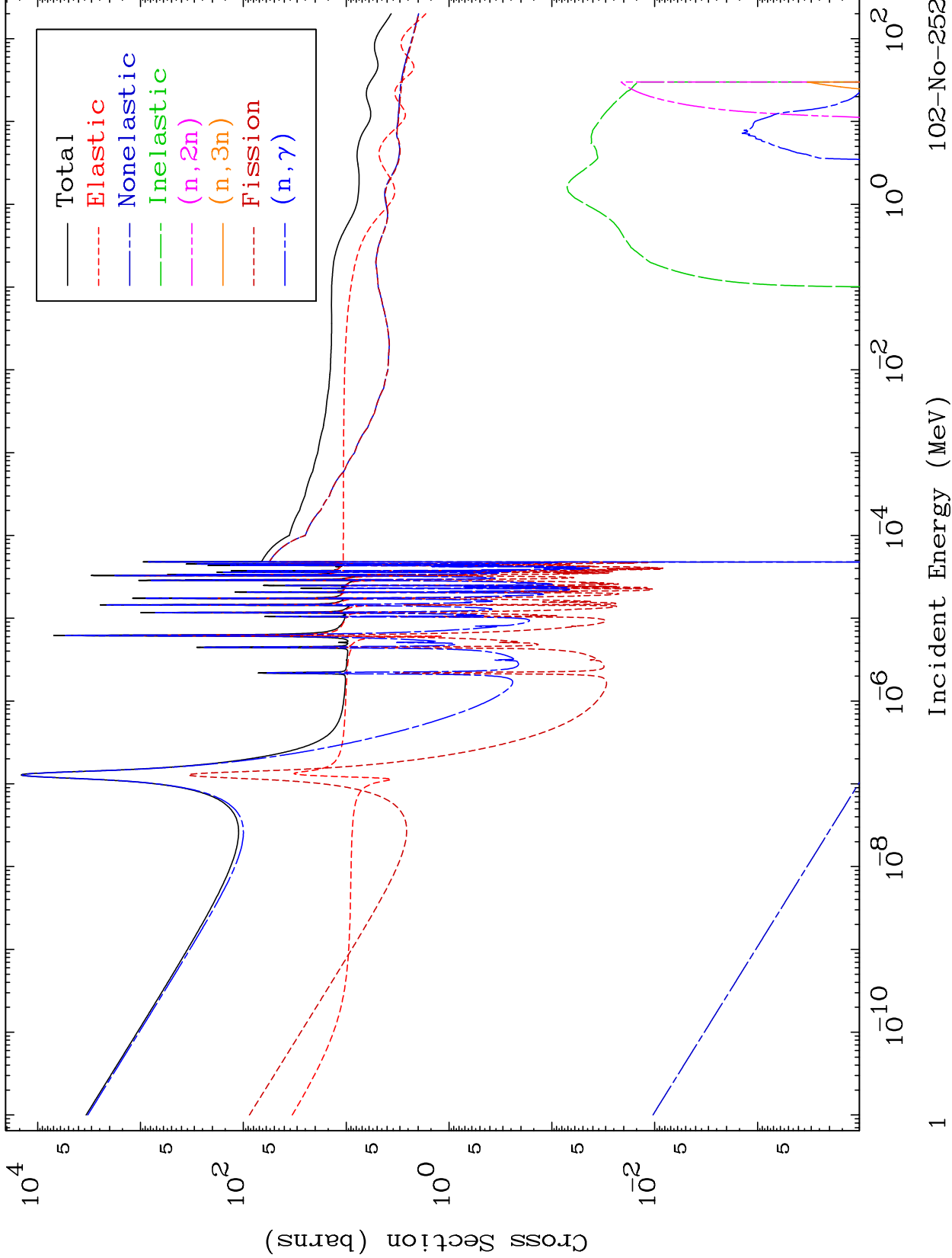
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Press Mouse Button to Start

MAT 9970

Neutron Major
293 Kelvin Cross Sections

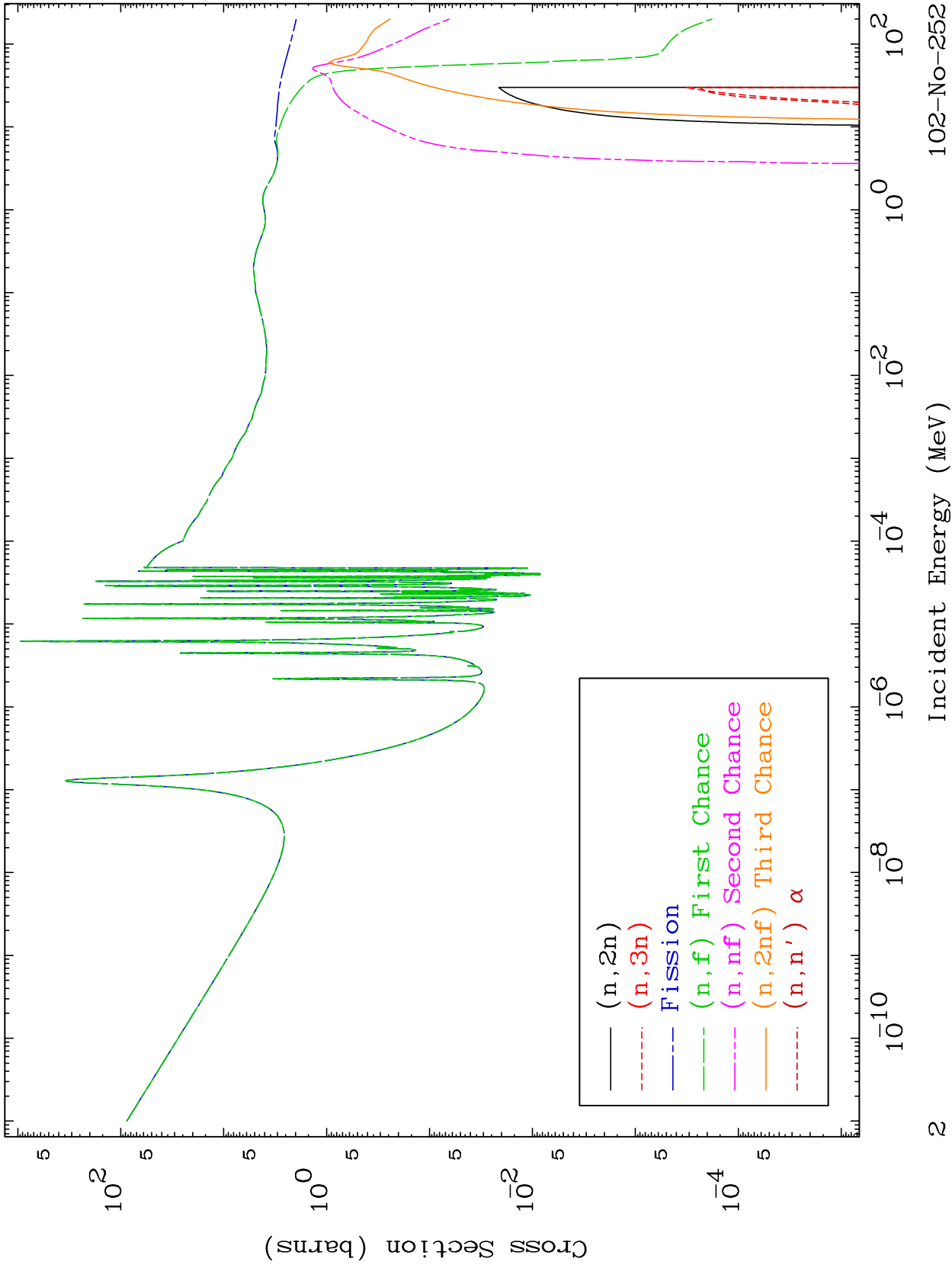
102-No-252



MAT 9970

Neutron Absorption
293 Kelvin Cross Sections

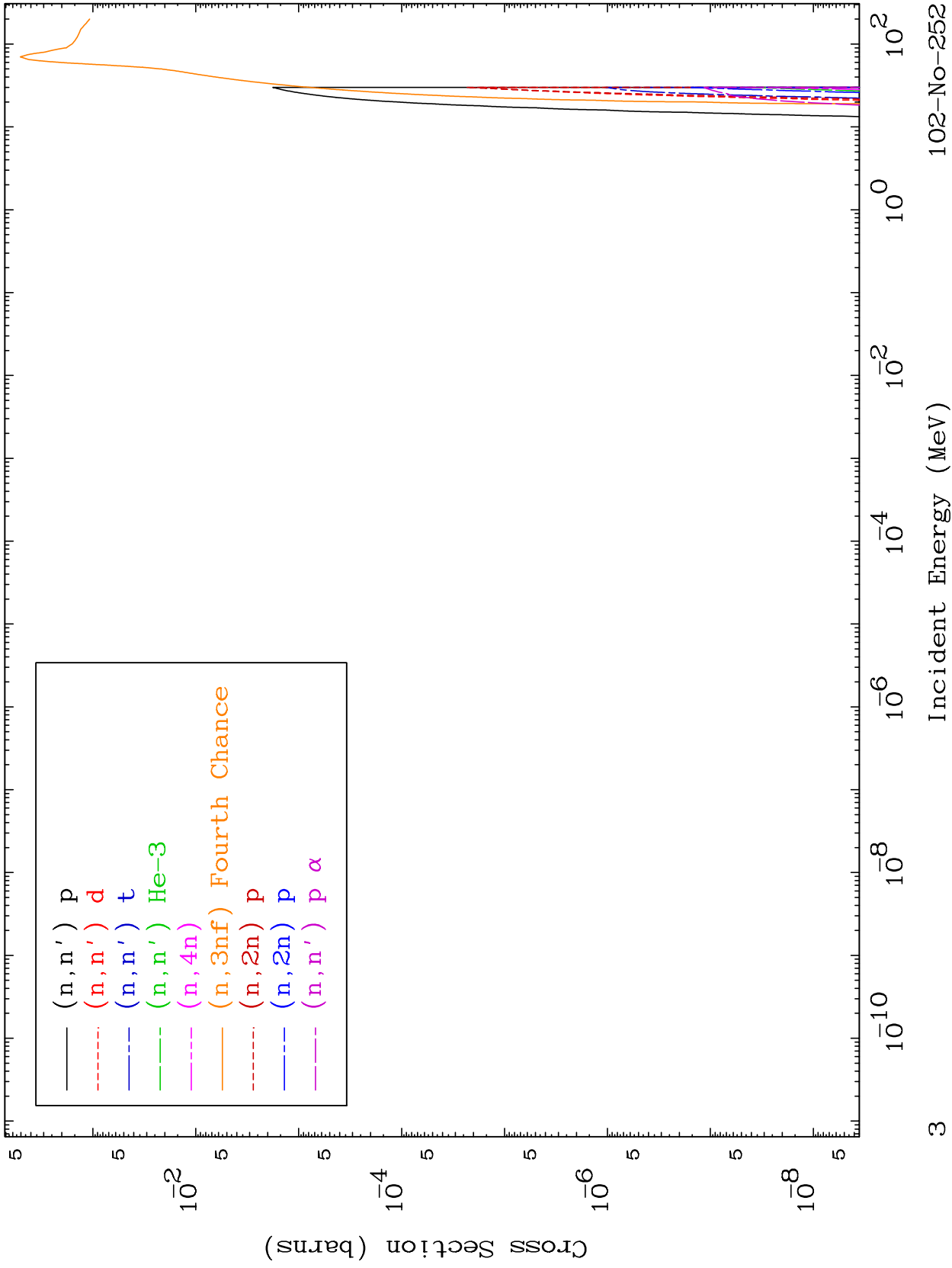
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MAT 9970

Neutron Absorption
293 Kelvin Cross Sections

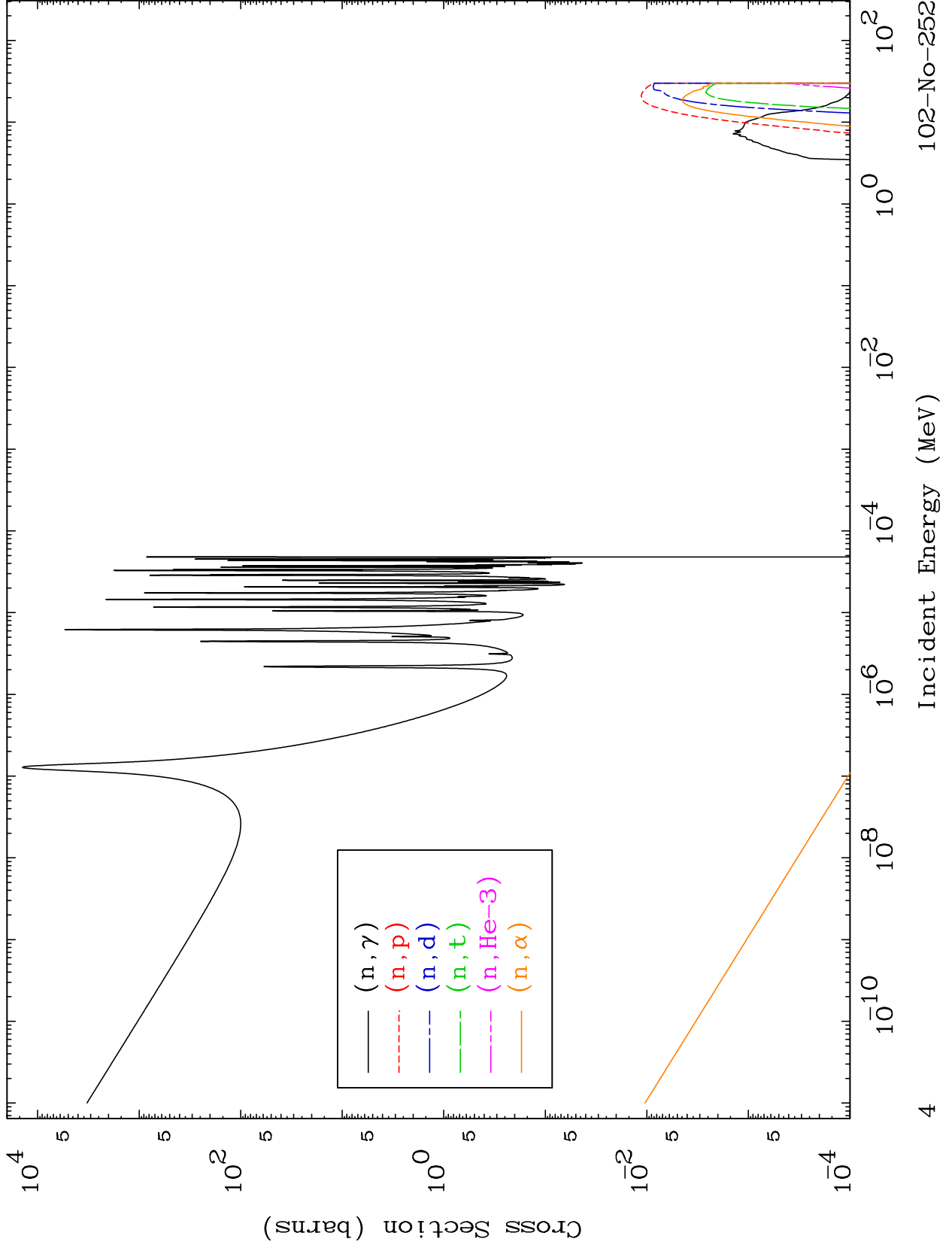
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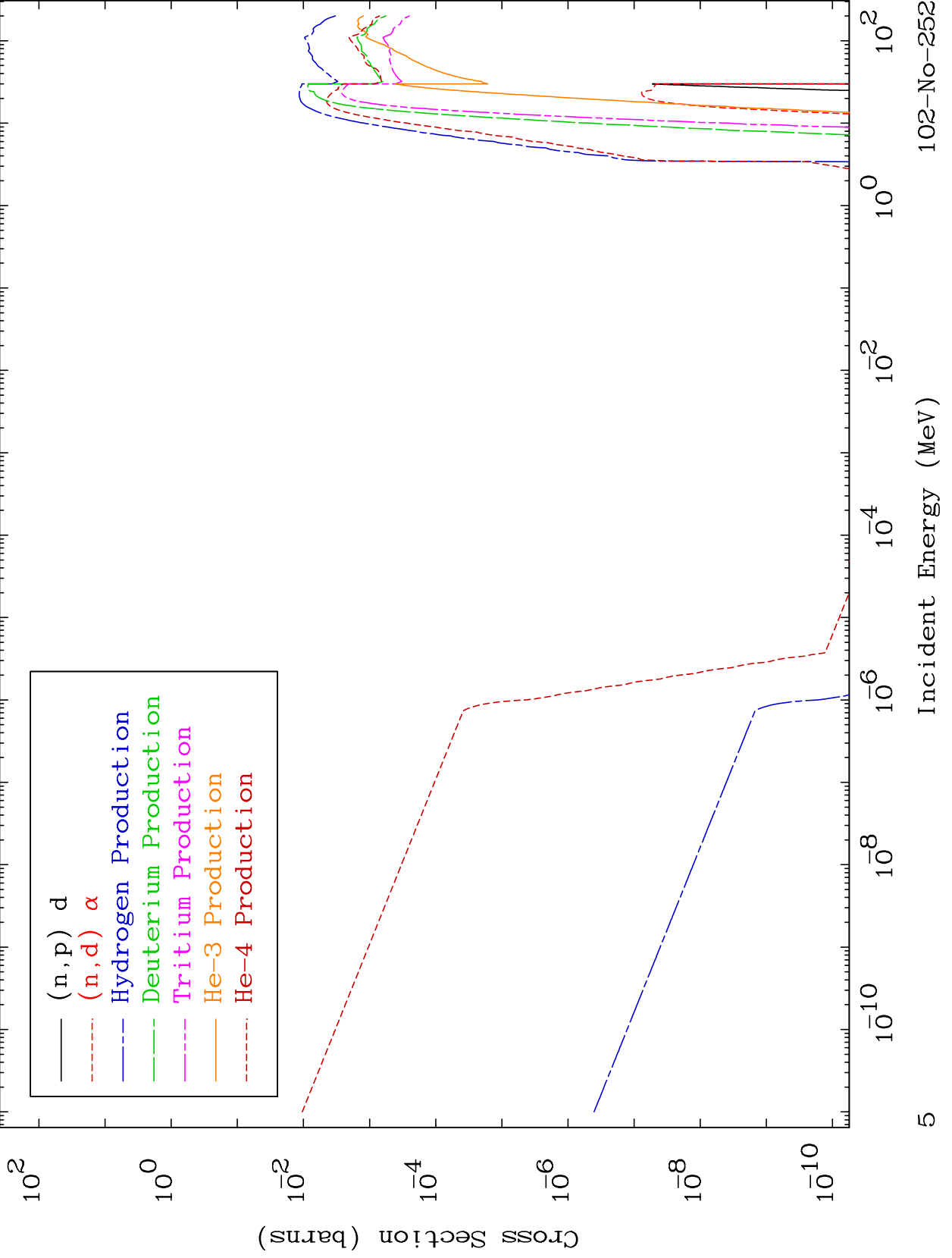


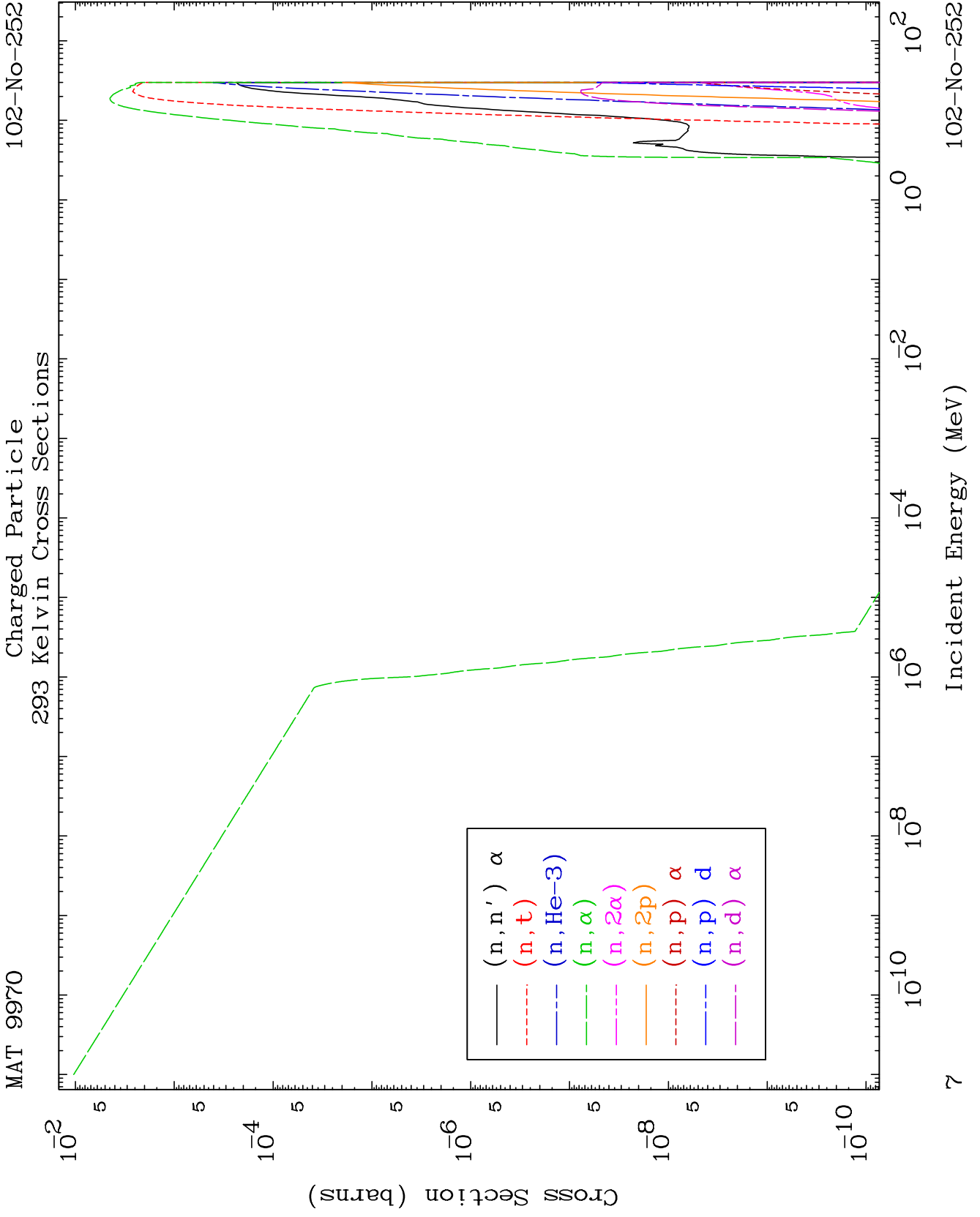
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Neutron Absorption
293 Kelvin Cross Sections

102-No-252



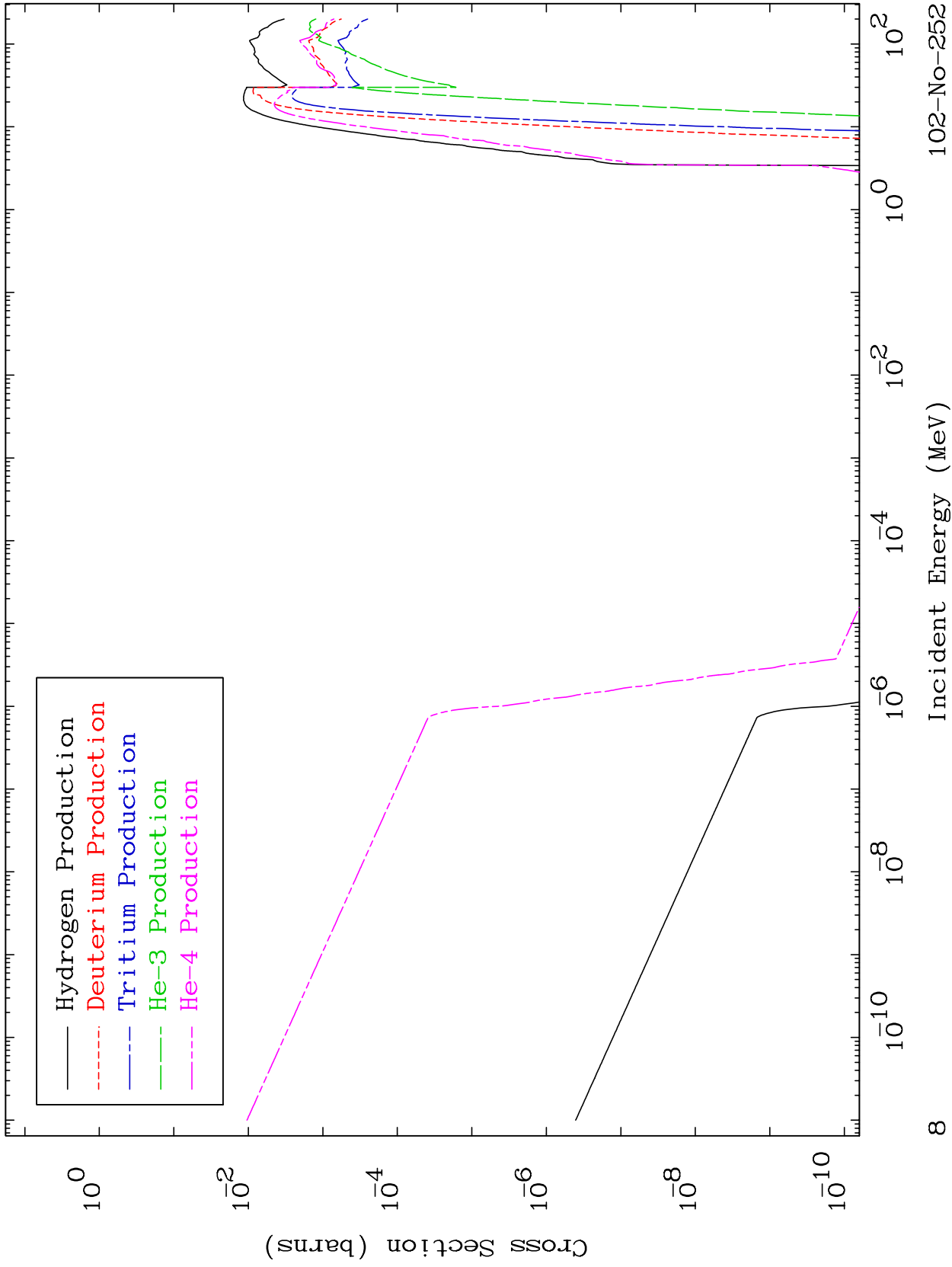




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Particle Production
293 Kelvin Cross Sections

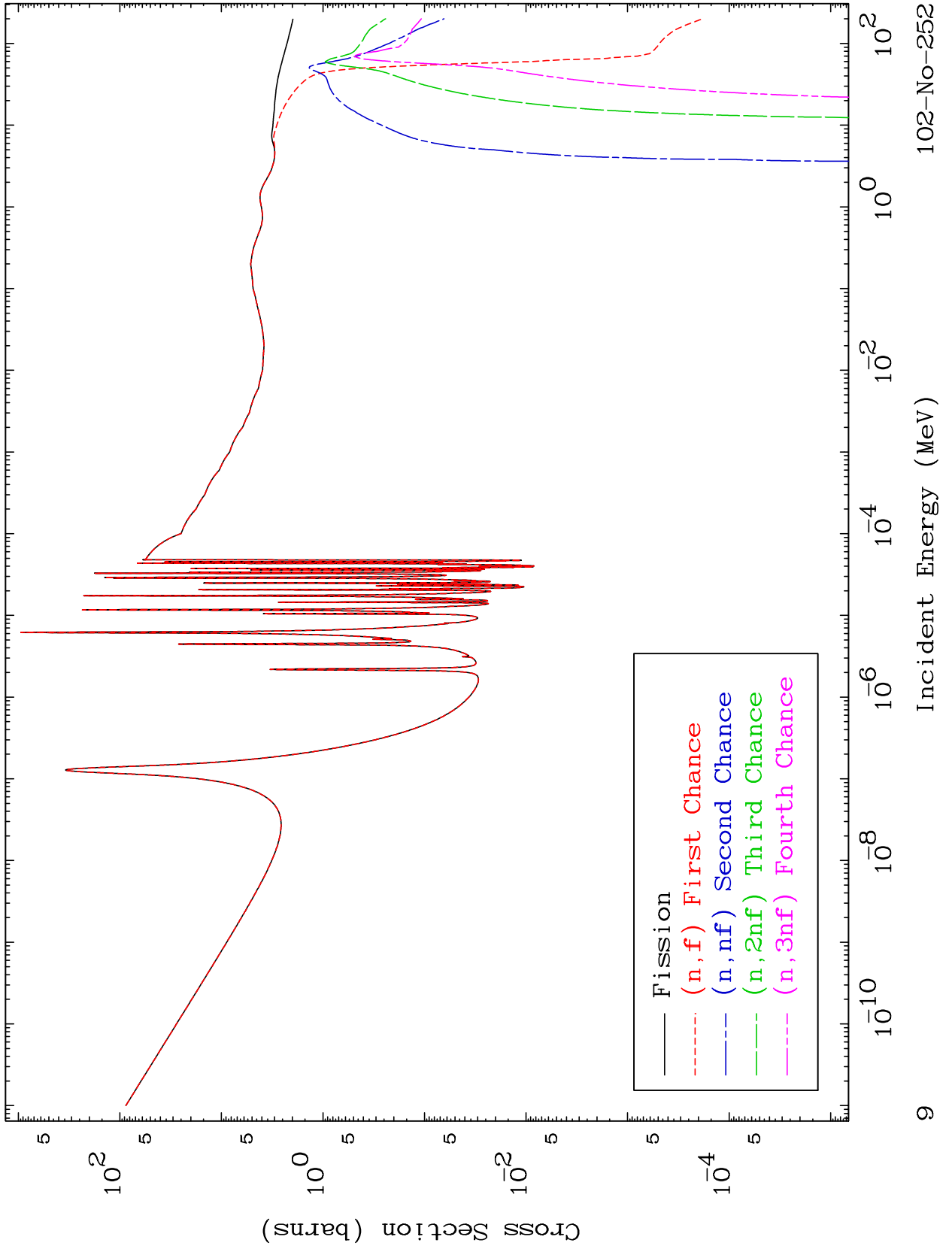
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Fission
293 Kelvin Cross Sections

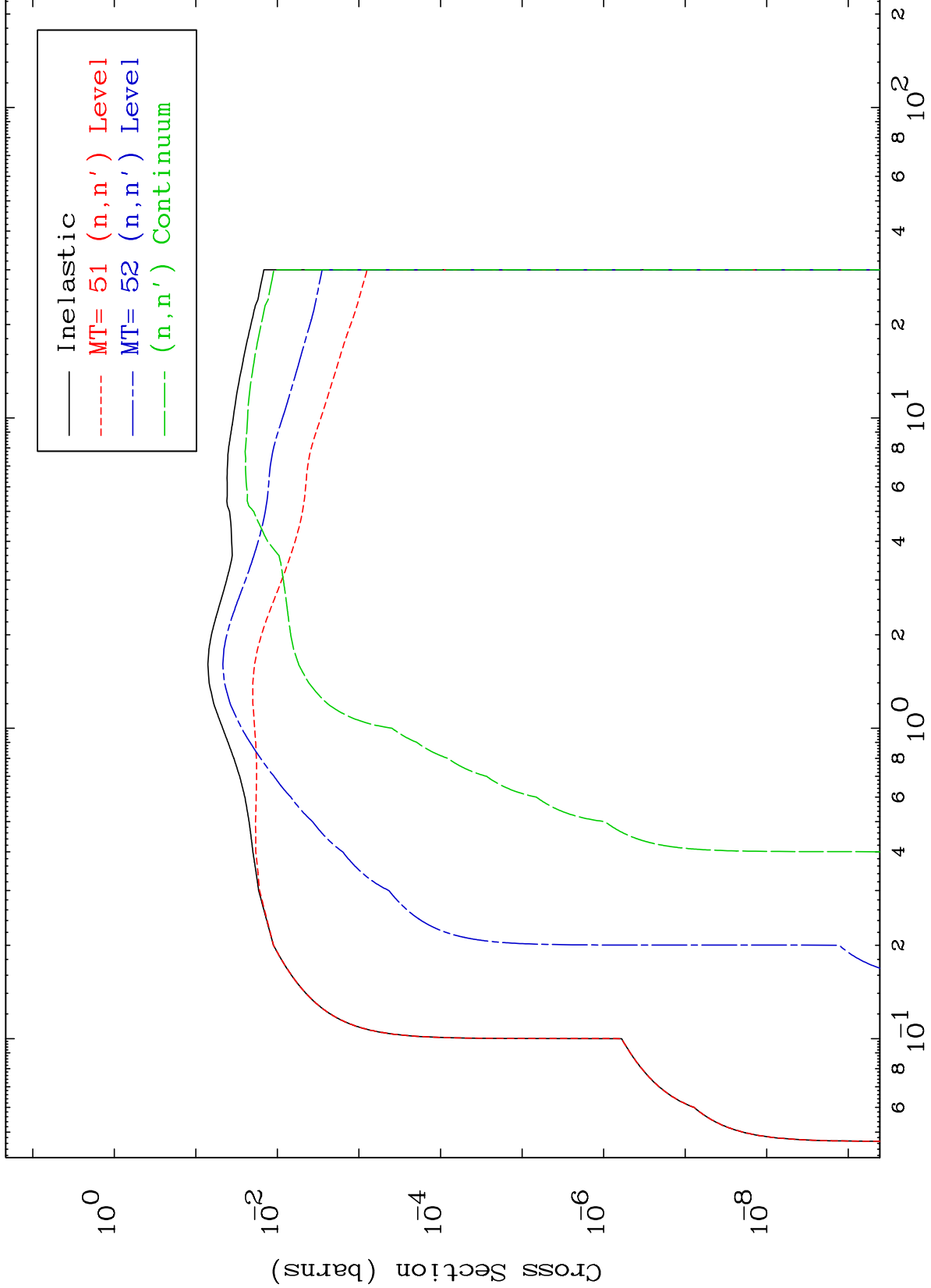
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(n,n') Levels
293 Kelvin Cross Sections

102-No-252



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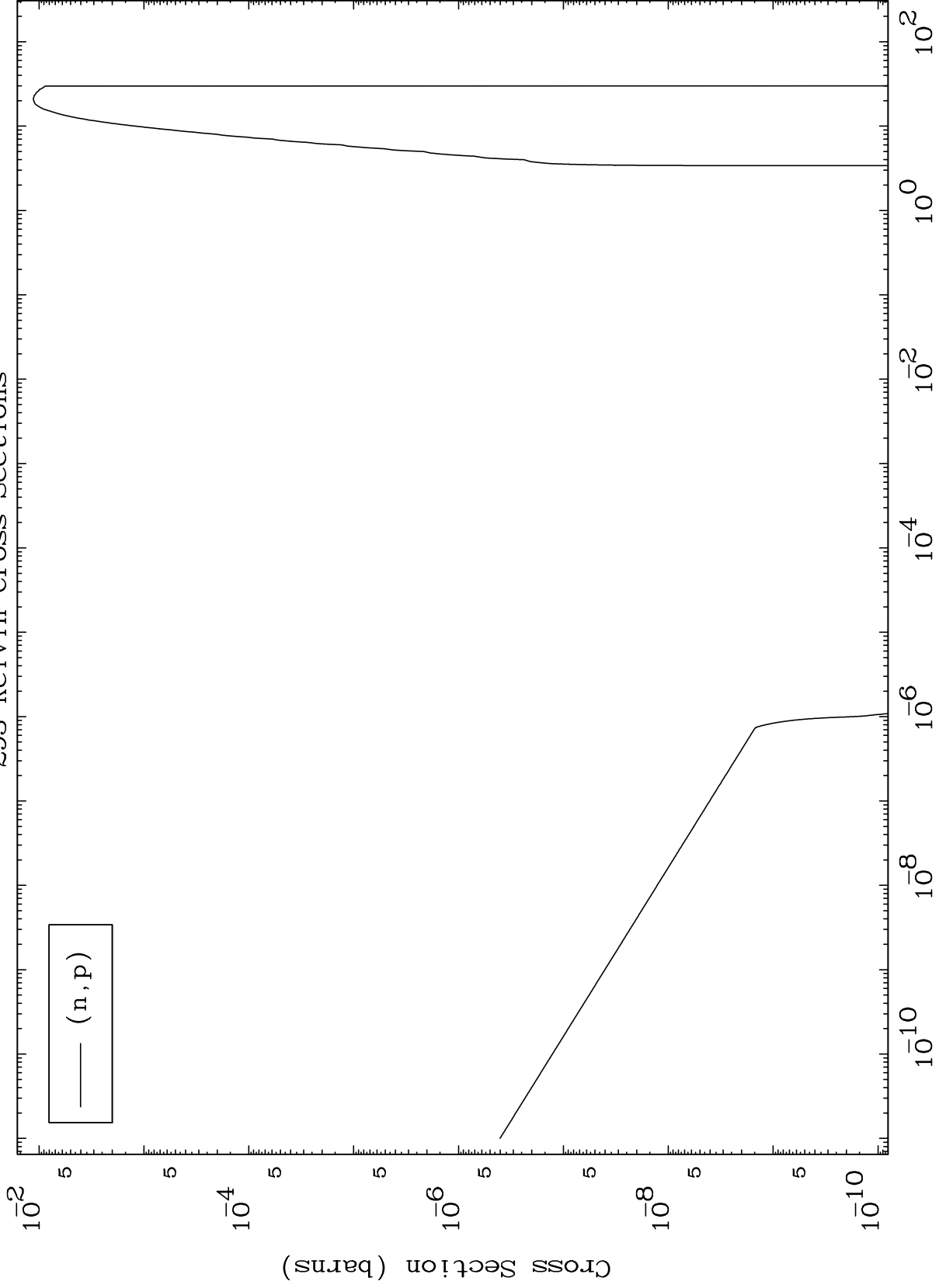
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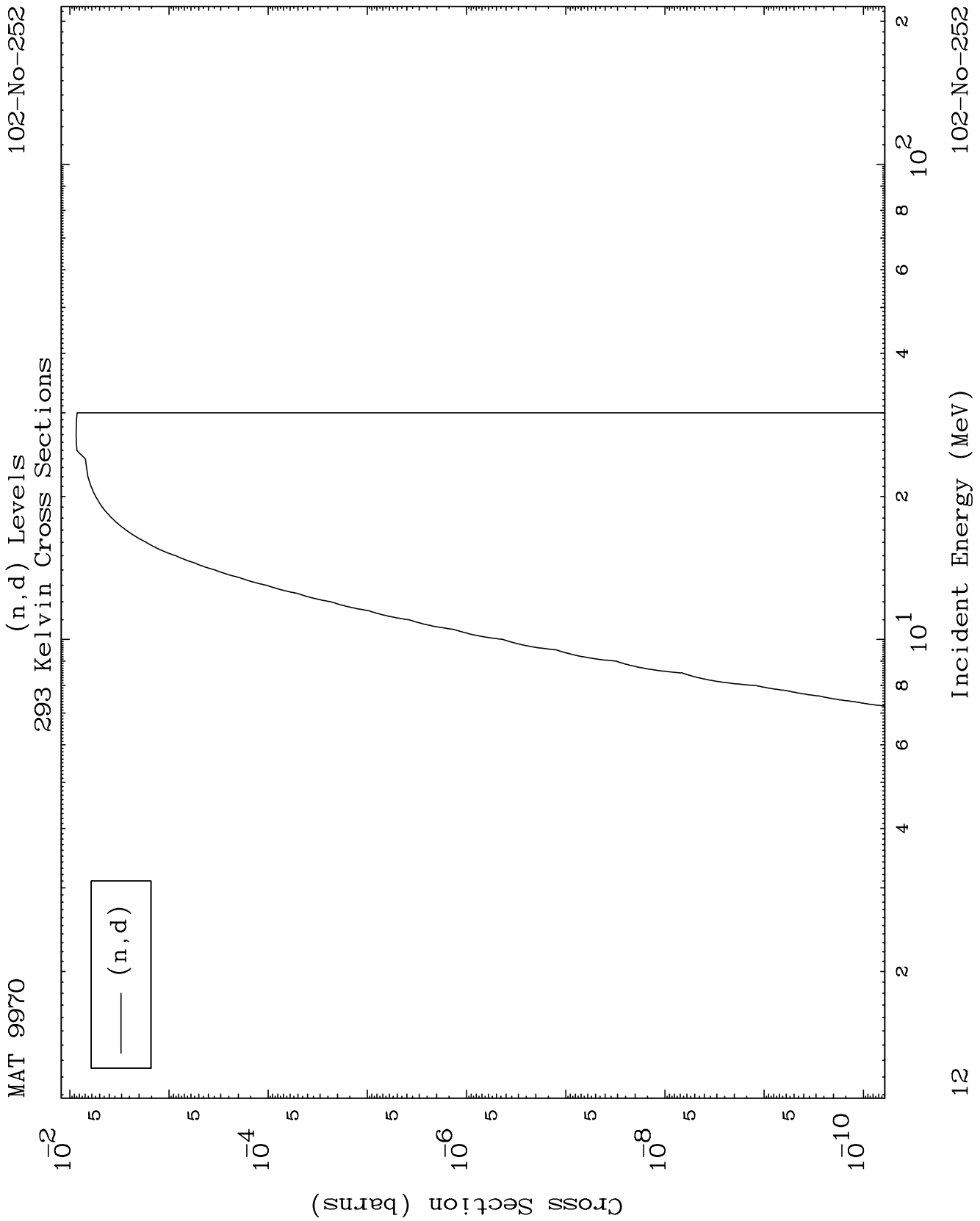
102-No-252

MAT 9970

(n,p) Levels
293 Kelvin Cross Sections

102-No-252

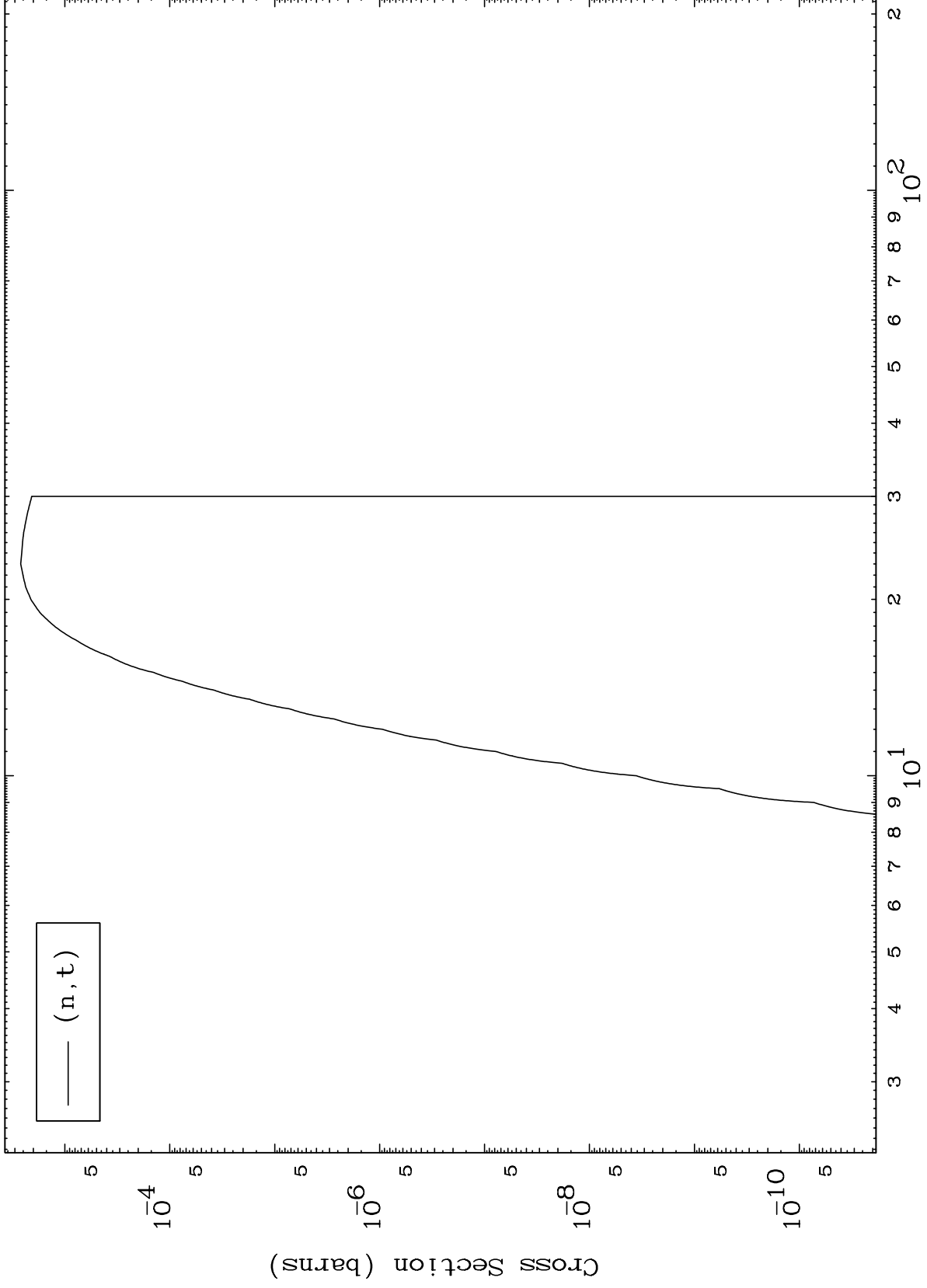




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(n,t) Levels
293 Kelvin Cross Sections

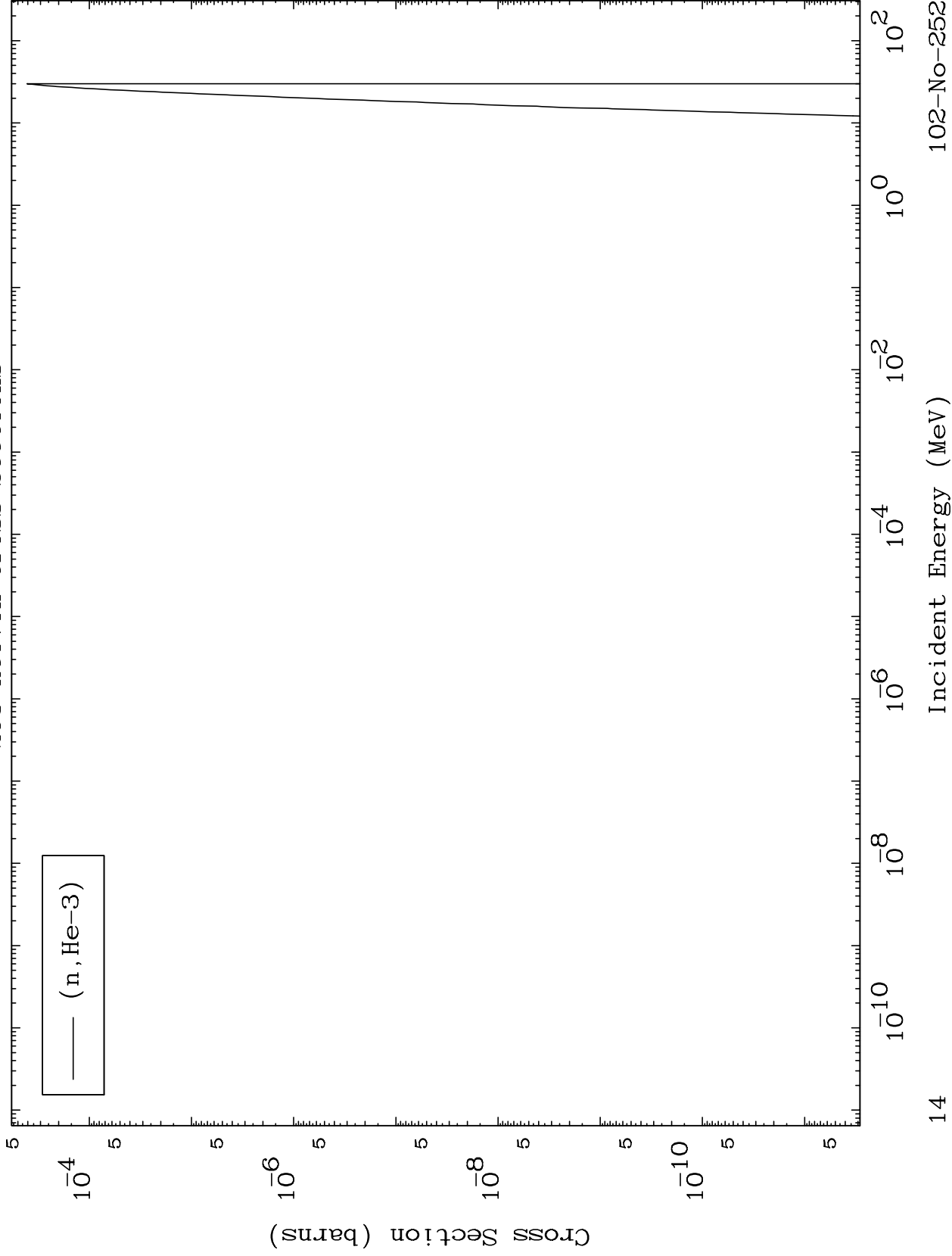
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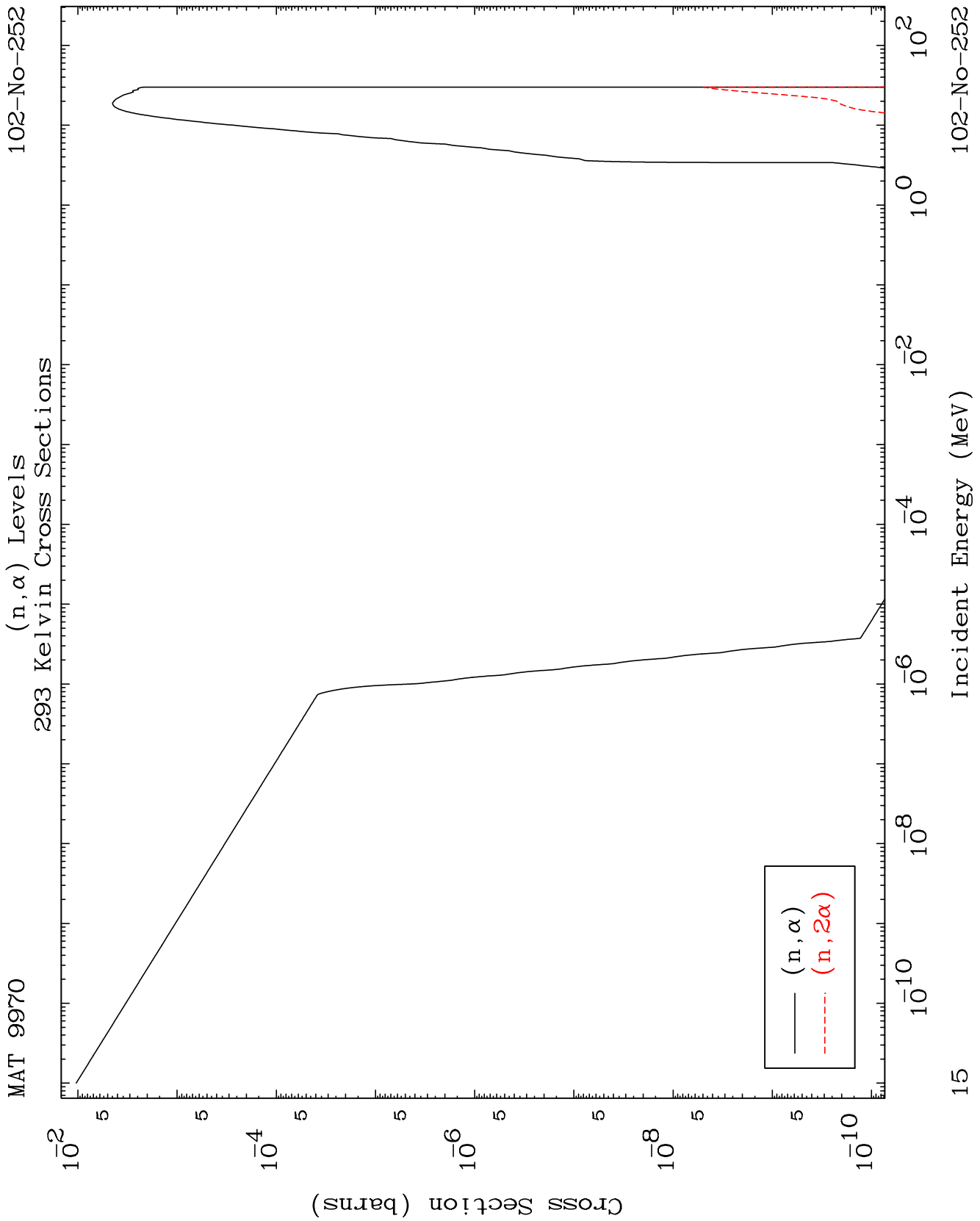


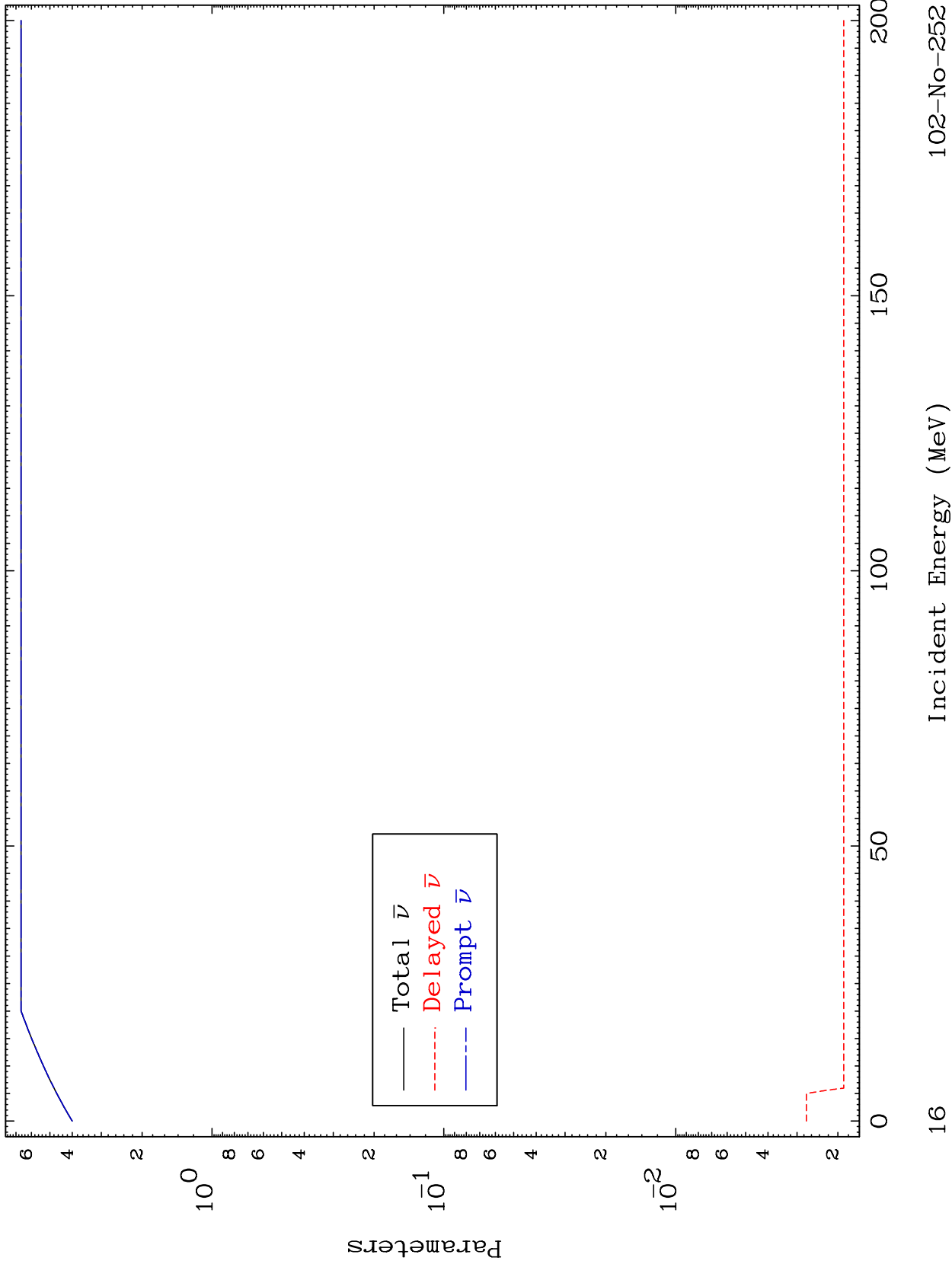
MAT 9970

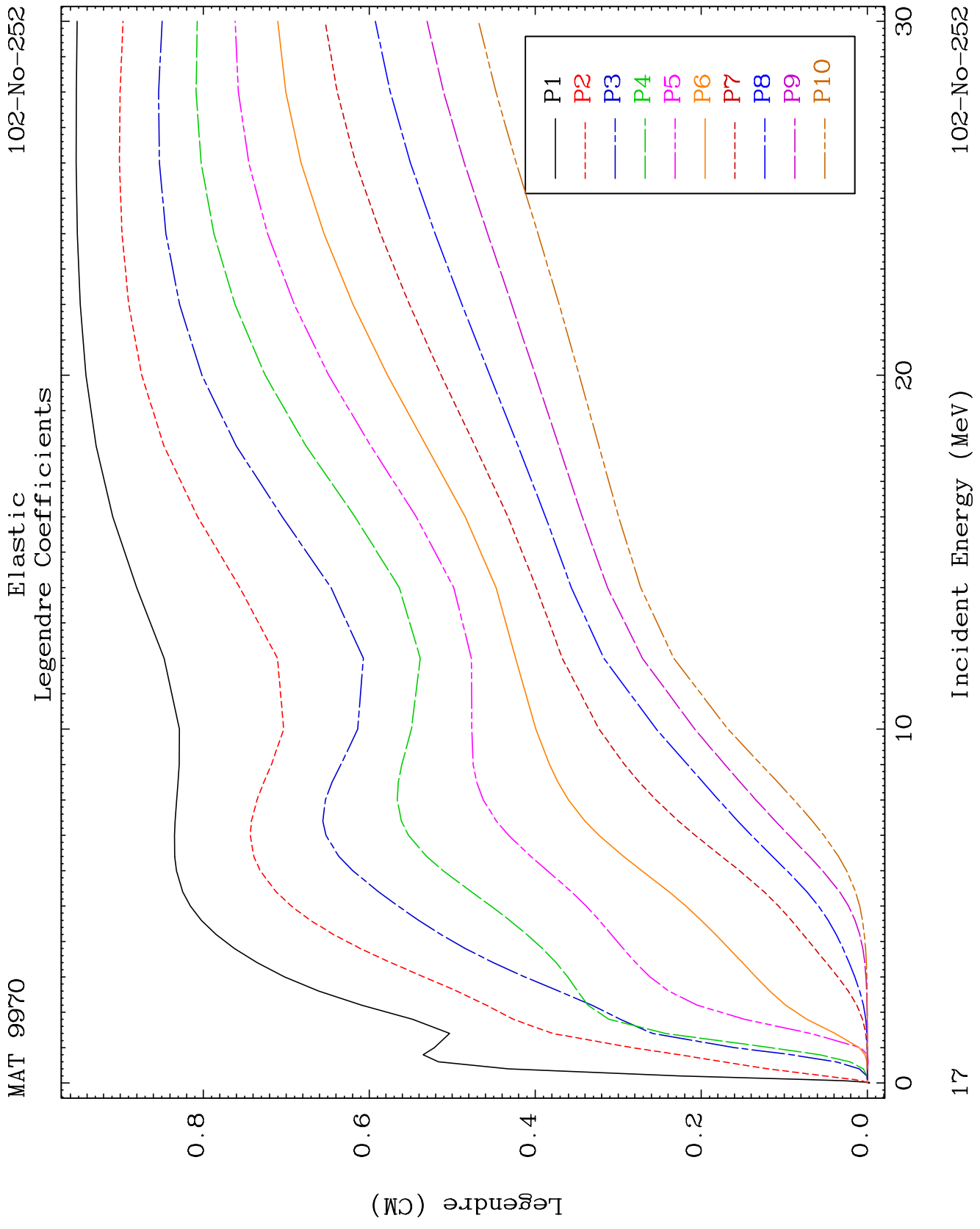
(n,He3) Levels
293 Kelvin Cross Sections

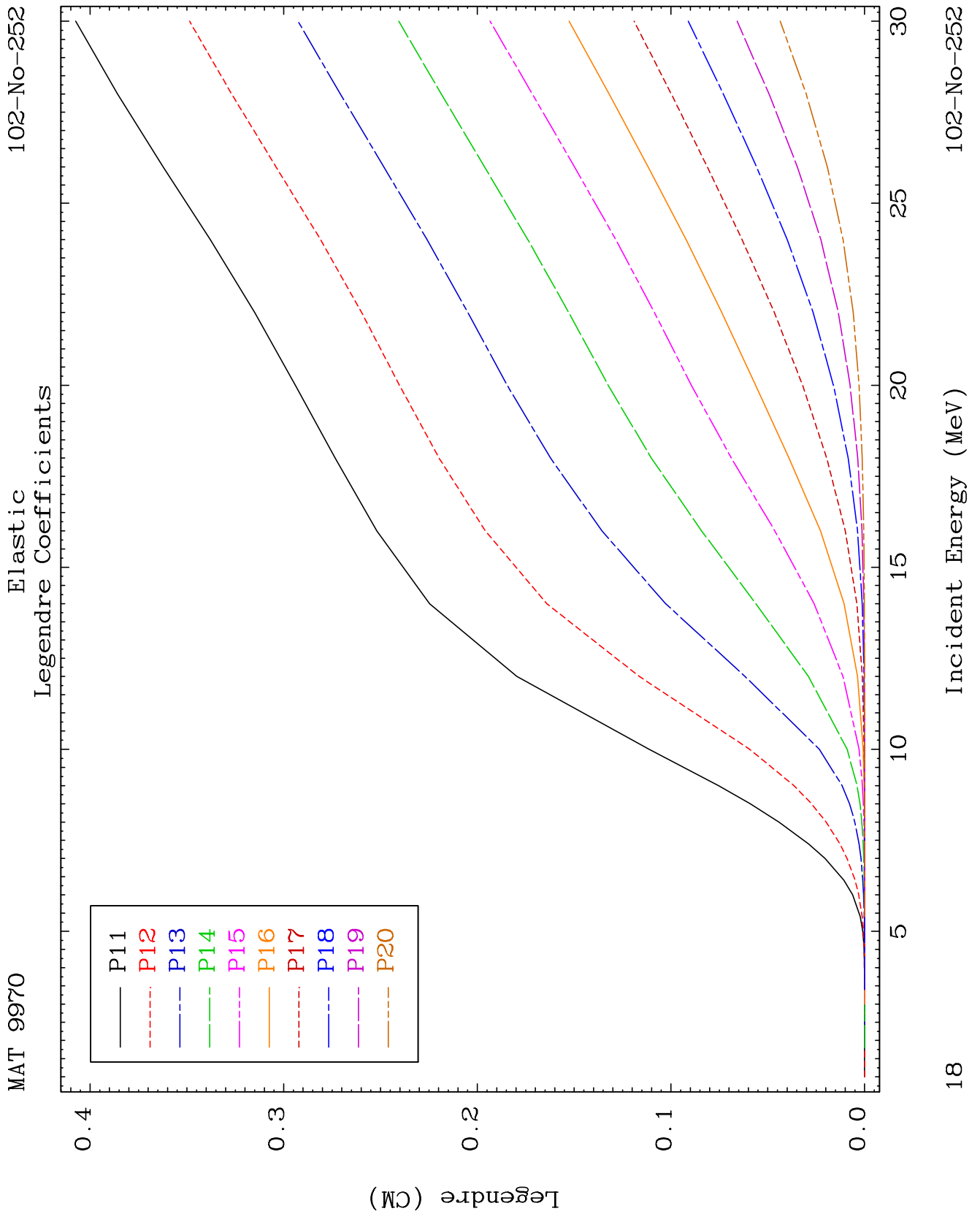
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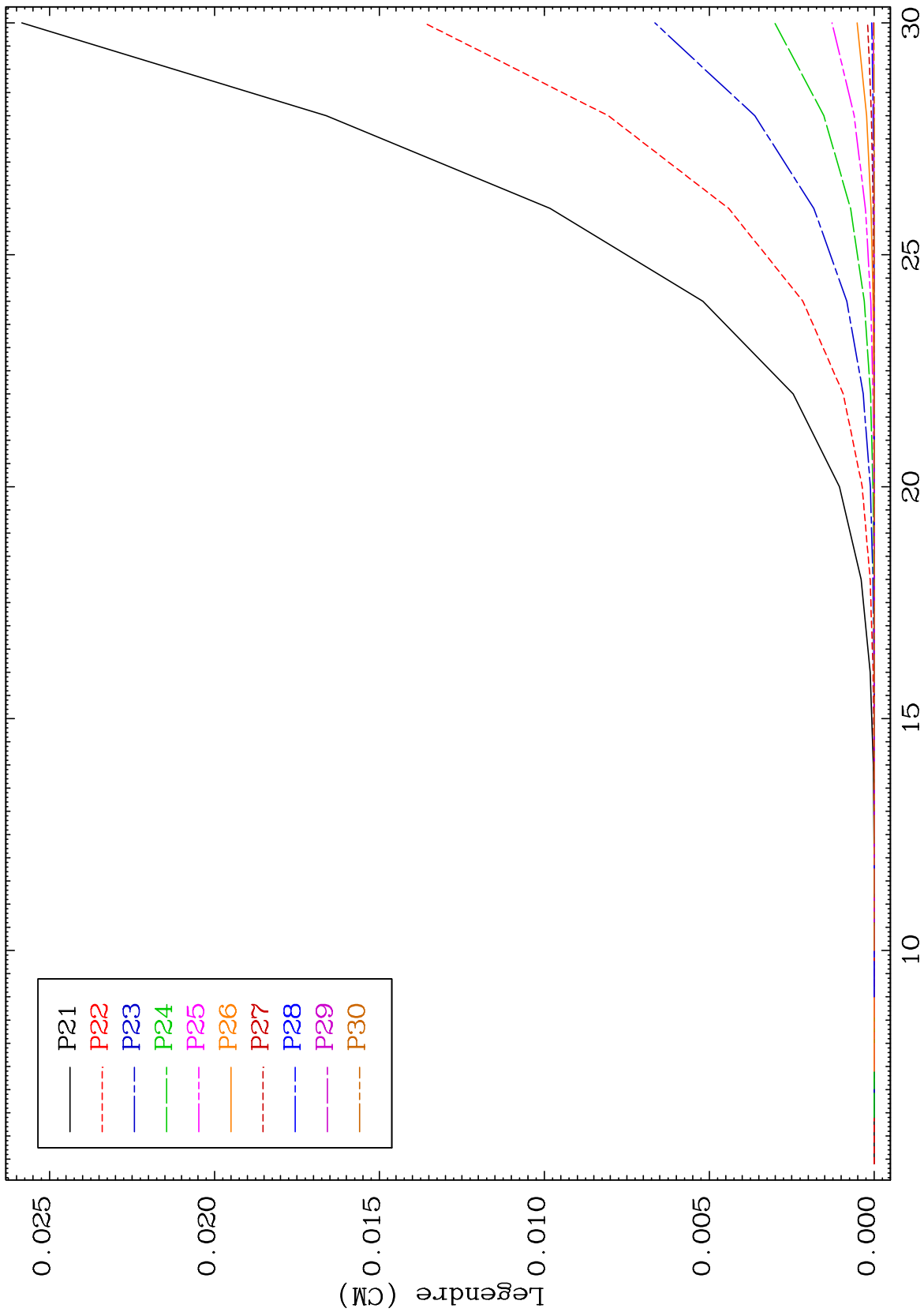




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Elastic Legendre Coefficients

102-No-252



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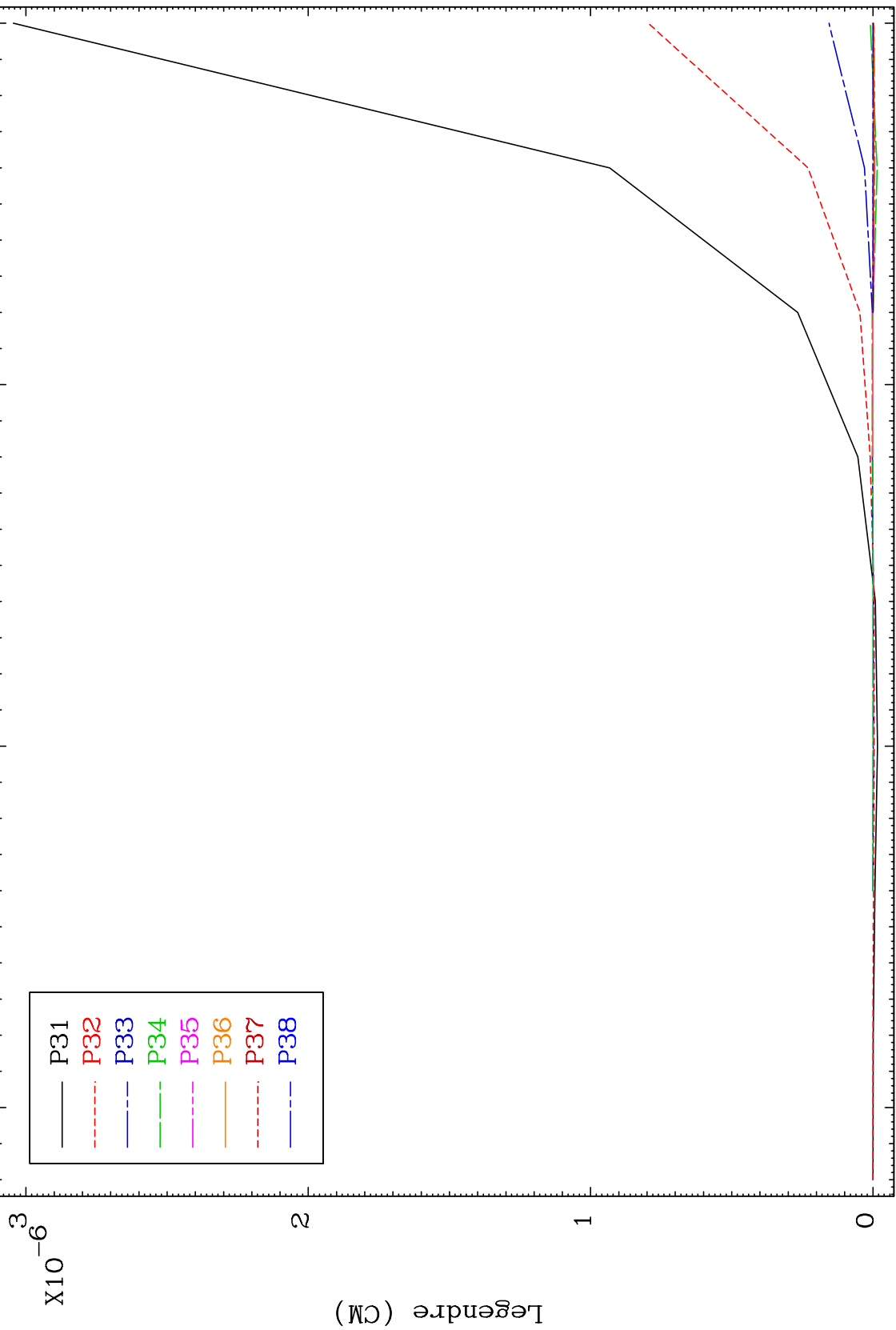
Incident Energy (MeV)

102-No-252

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Elastic Legendre Coefficients

102-No-252

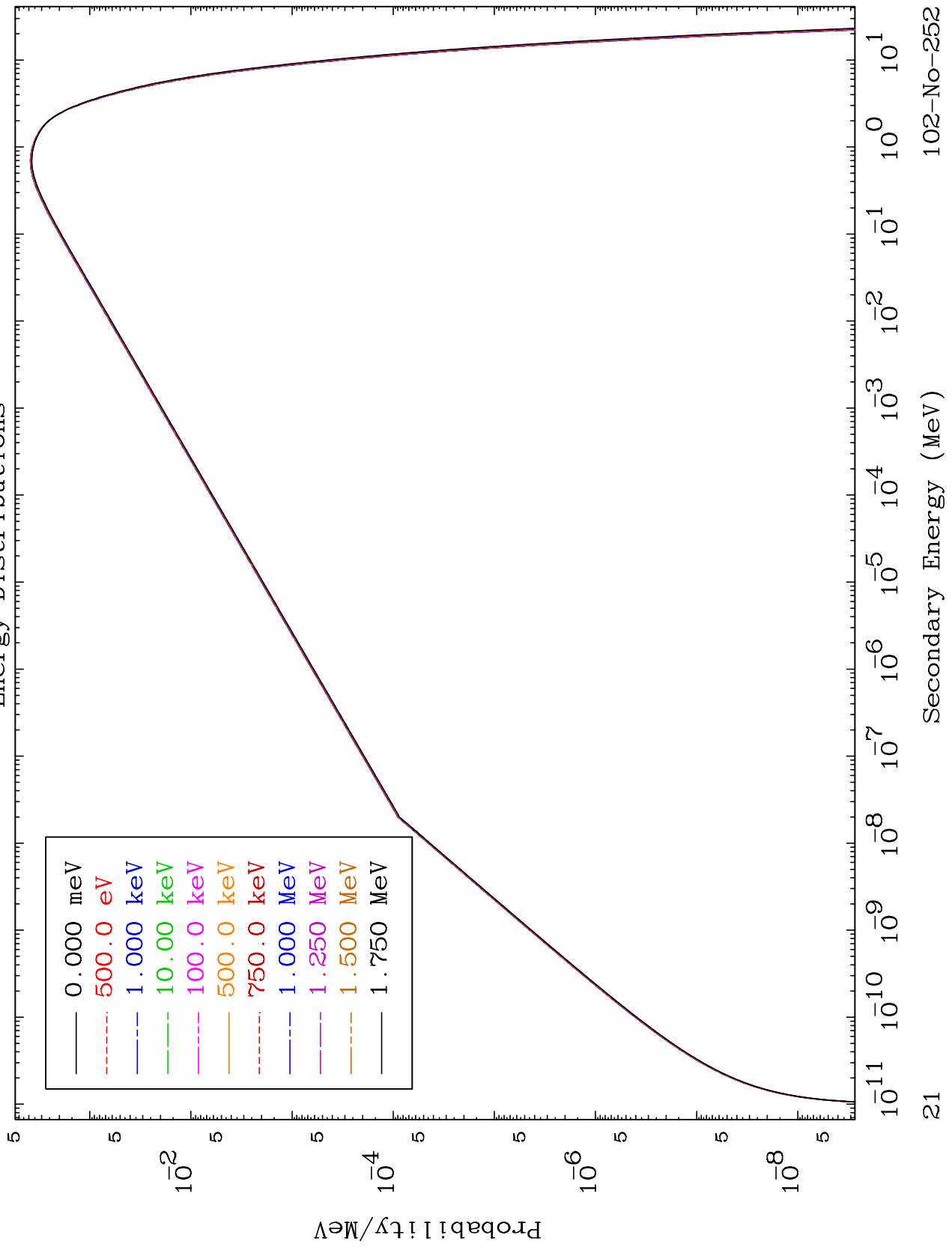


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Incident Energy (MeV)

102-No-252

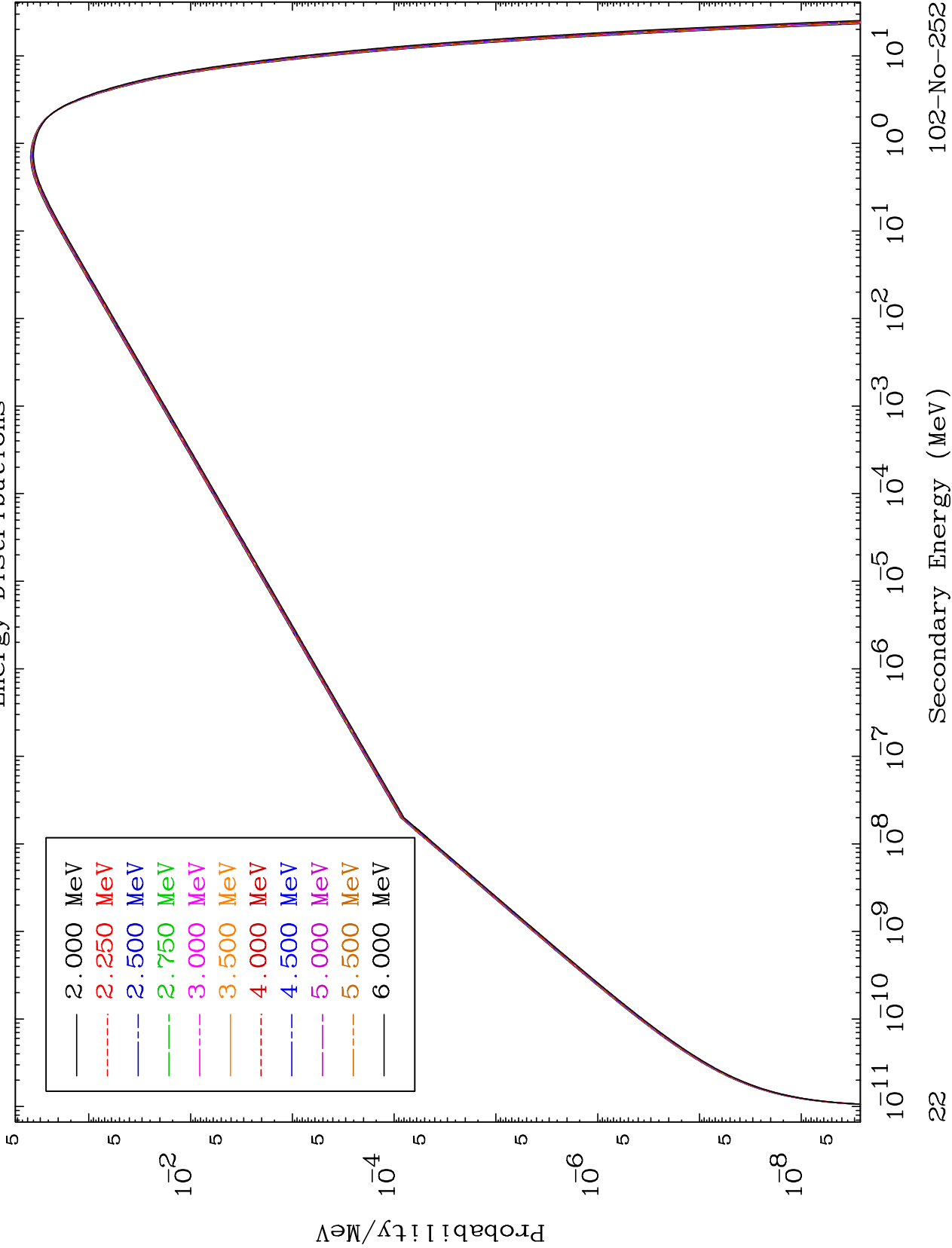
Fission Energy Distributions



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Fission Energy Distributions

102-No-252



102-No-252

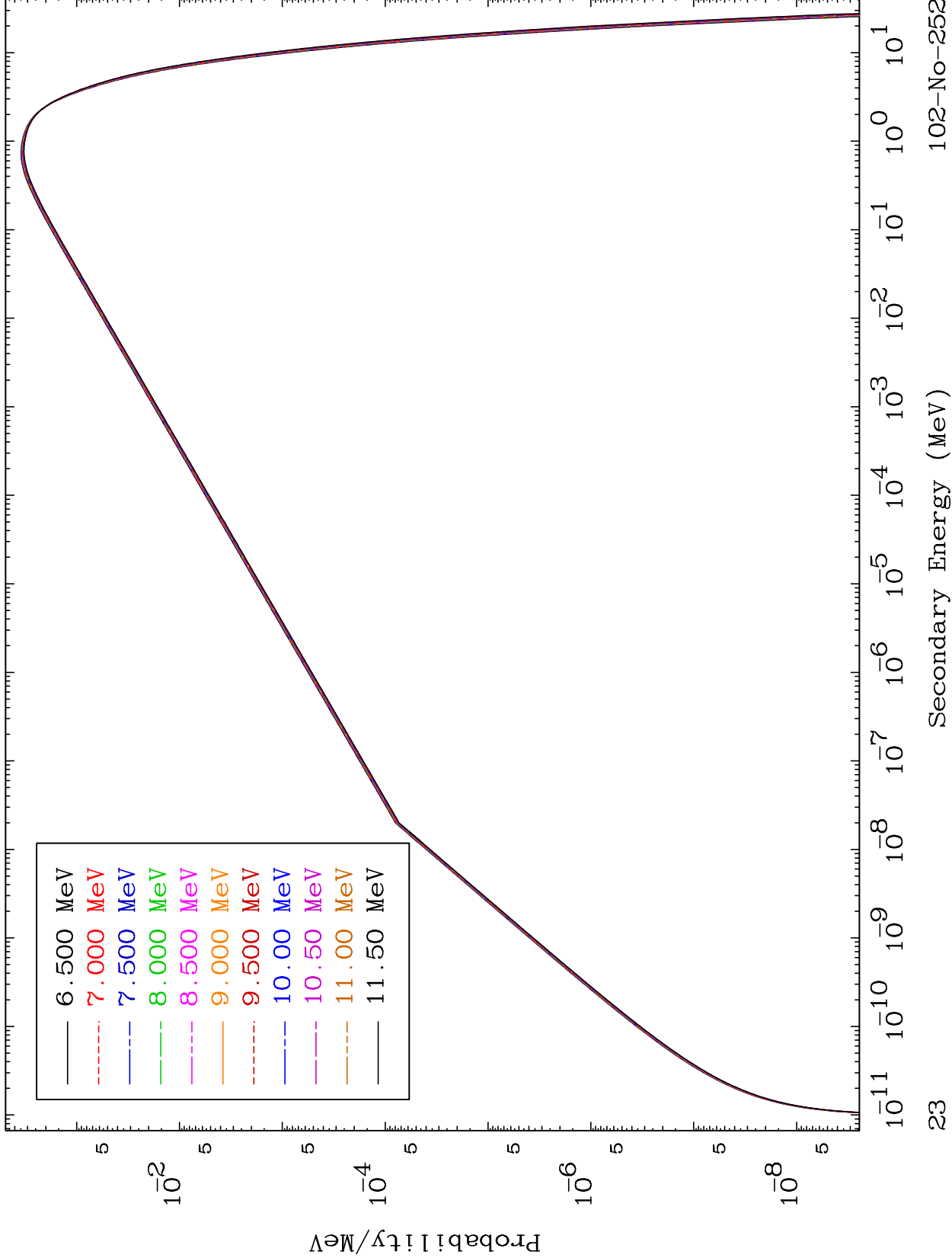
Secondary Energy (MeV)

22

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Fission Energy Distributions

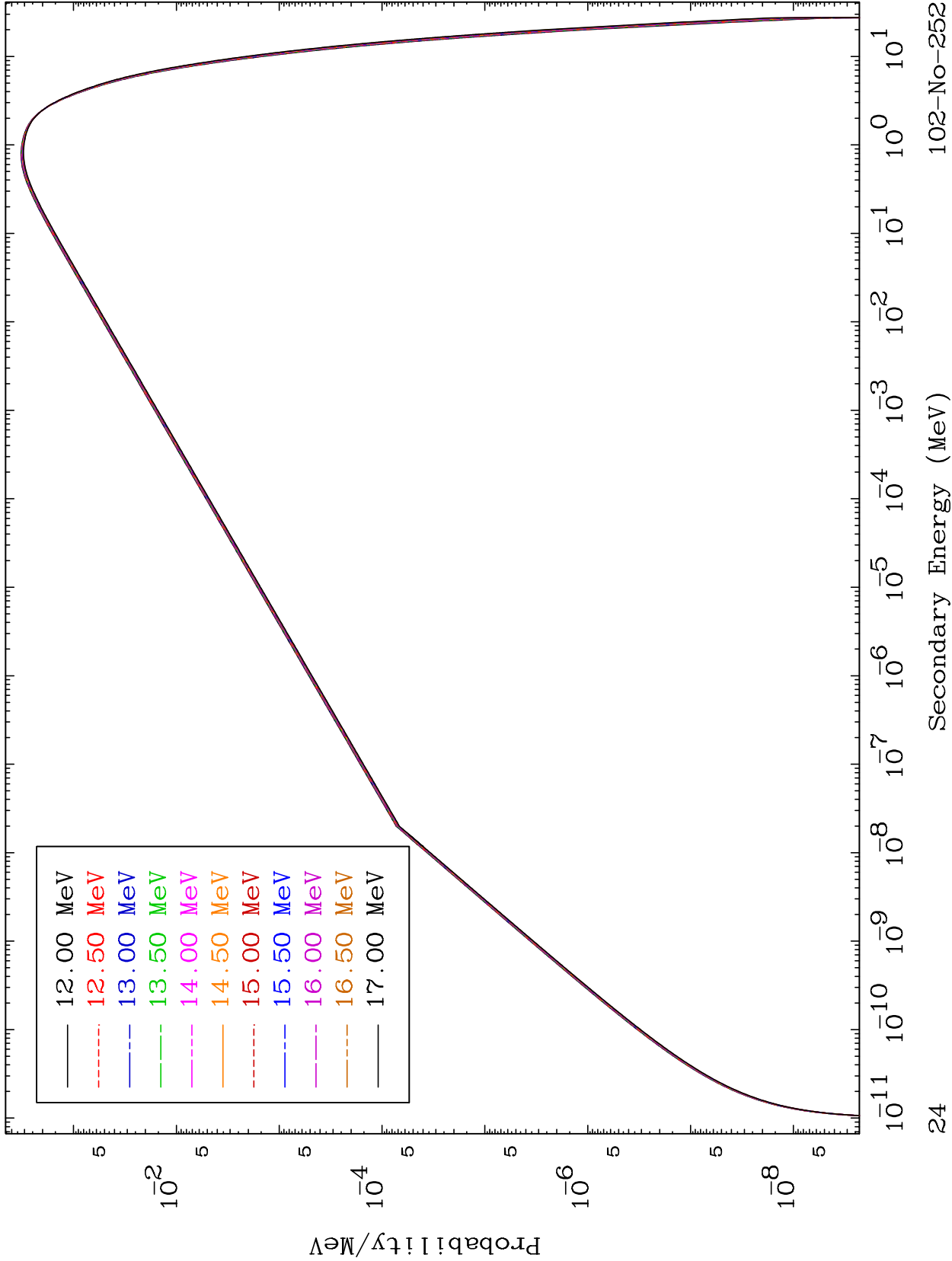
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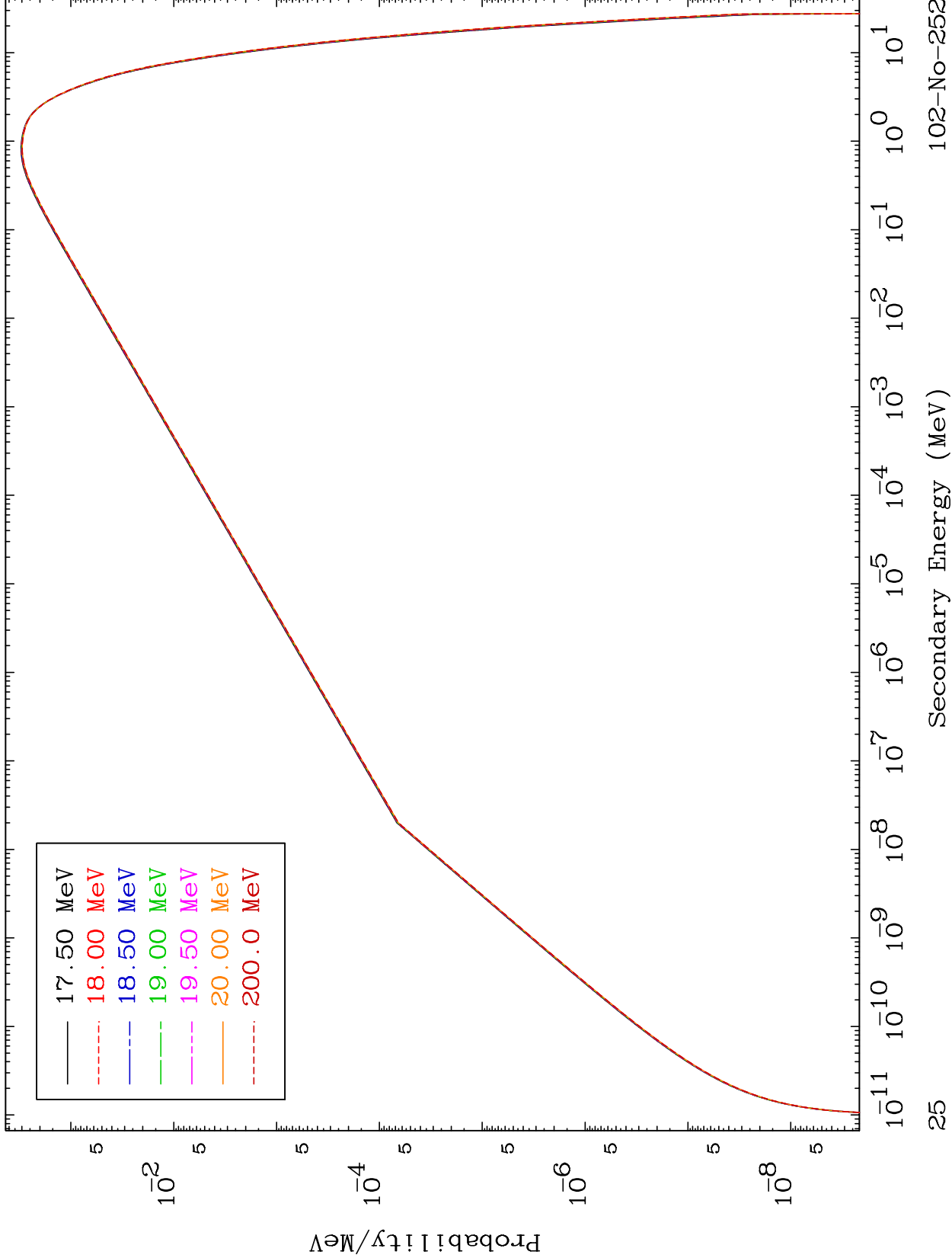
102-No-252

Secondary Energy (MeV)

Fission Energy Distributions



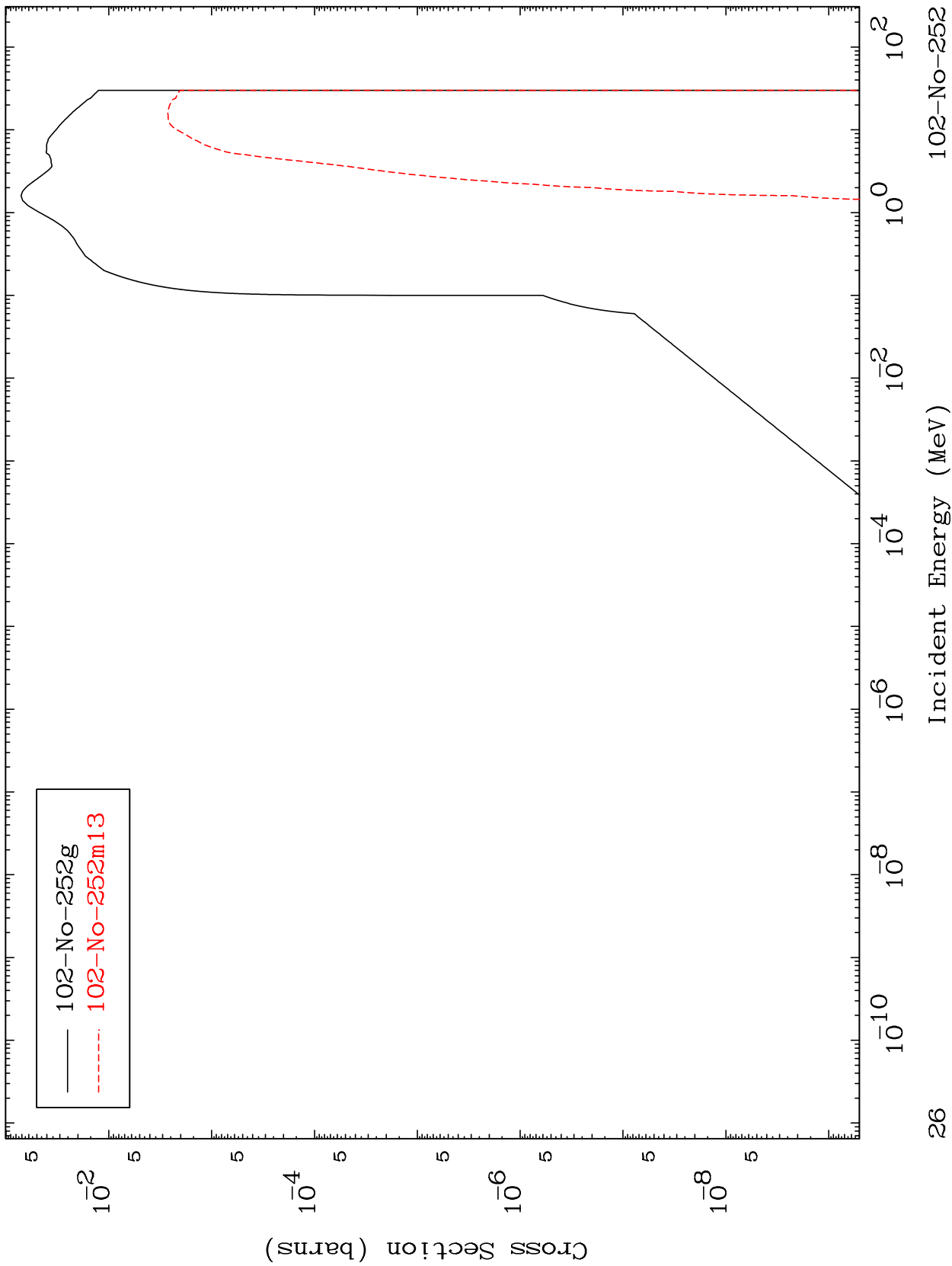
Fission Energy Distributions



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Radionuclide Production Cross Section

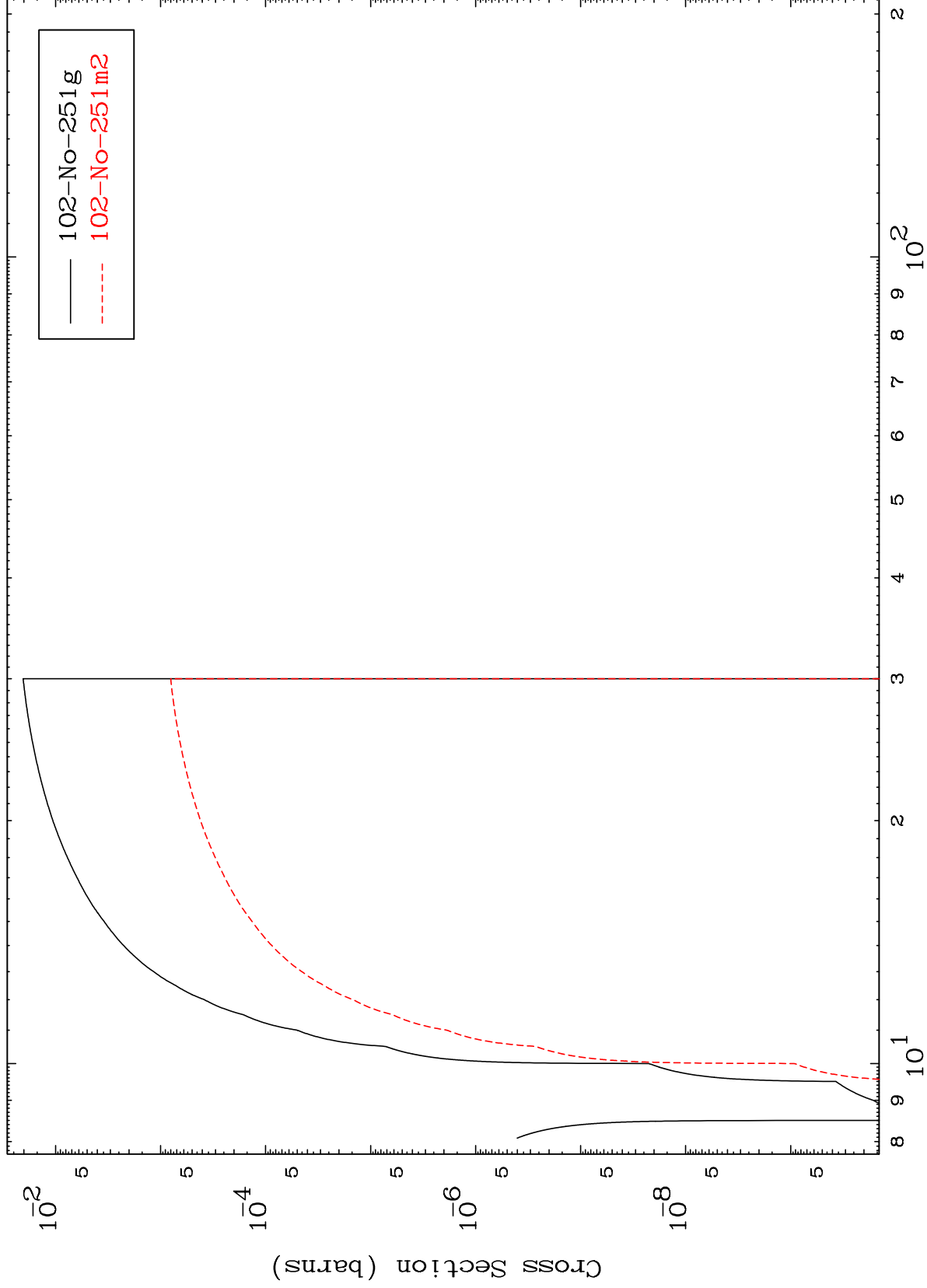


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(n,2n)

102-No-252

Radionuclide Production Cross Section



27

Incident Energy (MeV)

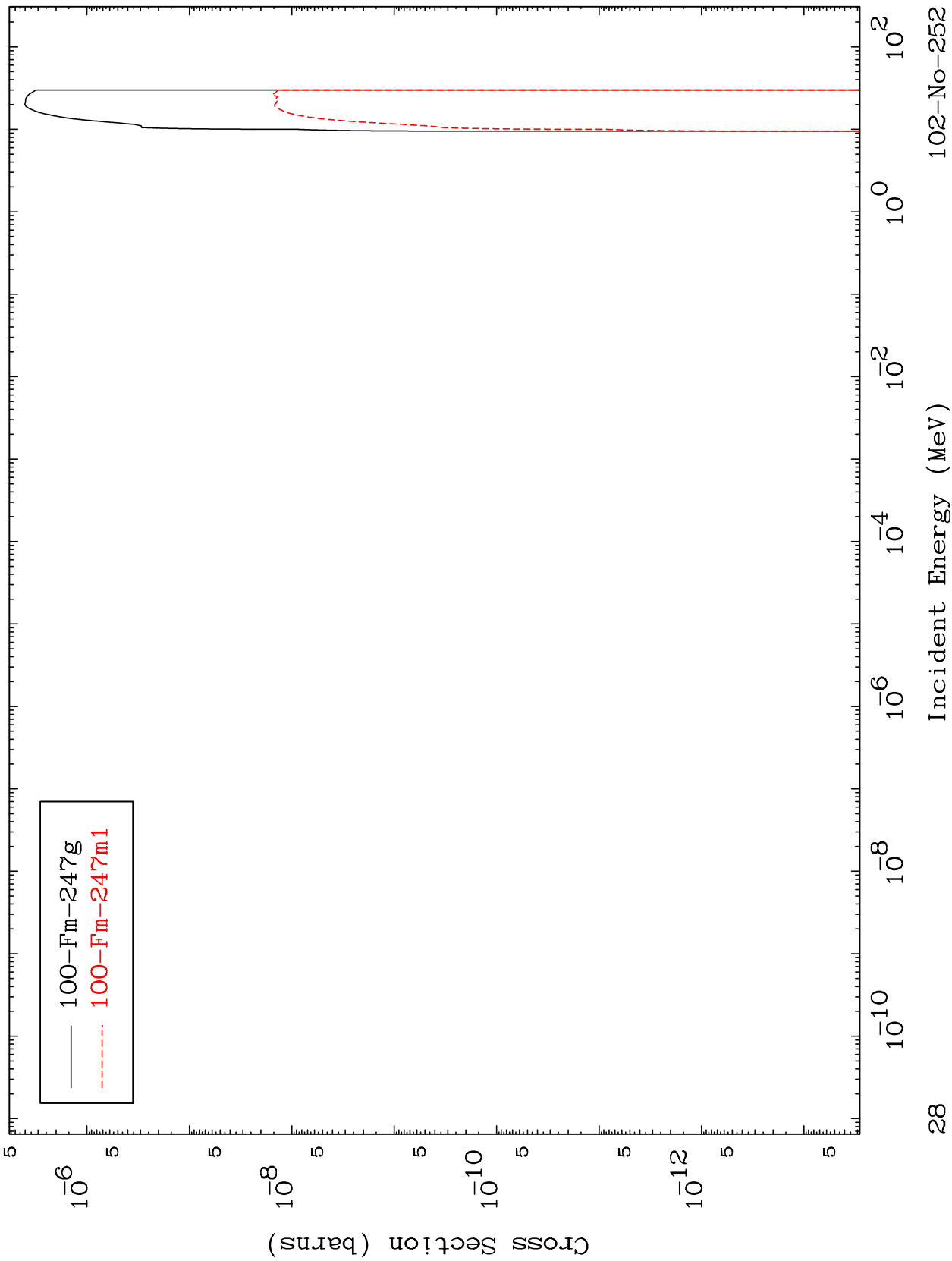
102-No-252

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(n,2n) α

102-No-252

Radionuclide Production Cross Section

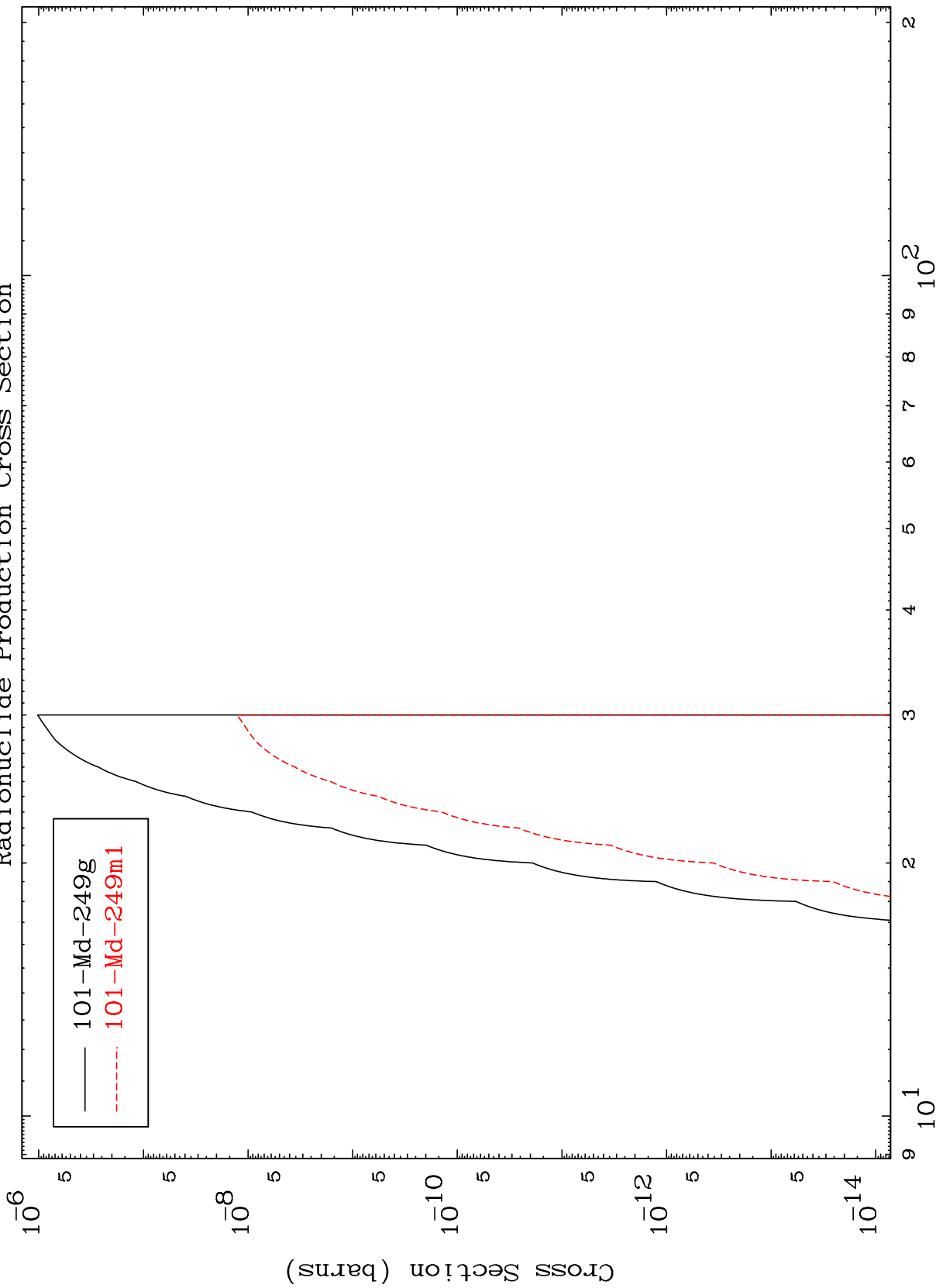


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(n,n') t

102-No-252

Radionuclide Production Cross Section



101-Md-249g
101-Md-249m1

29

Incident Energy (MeV)

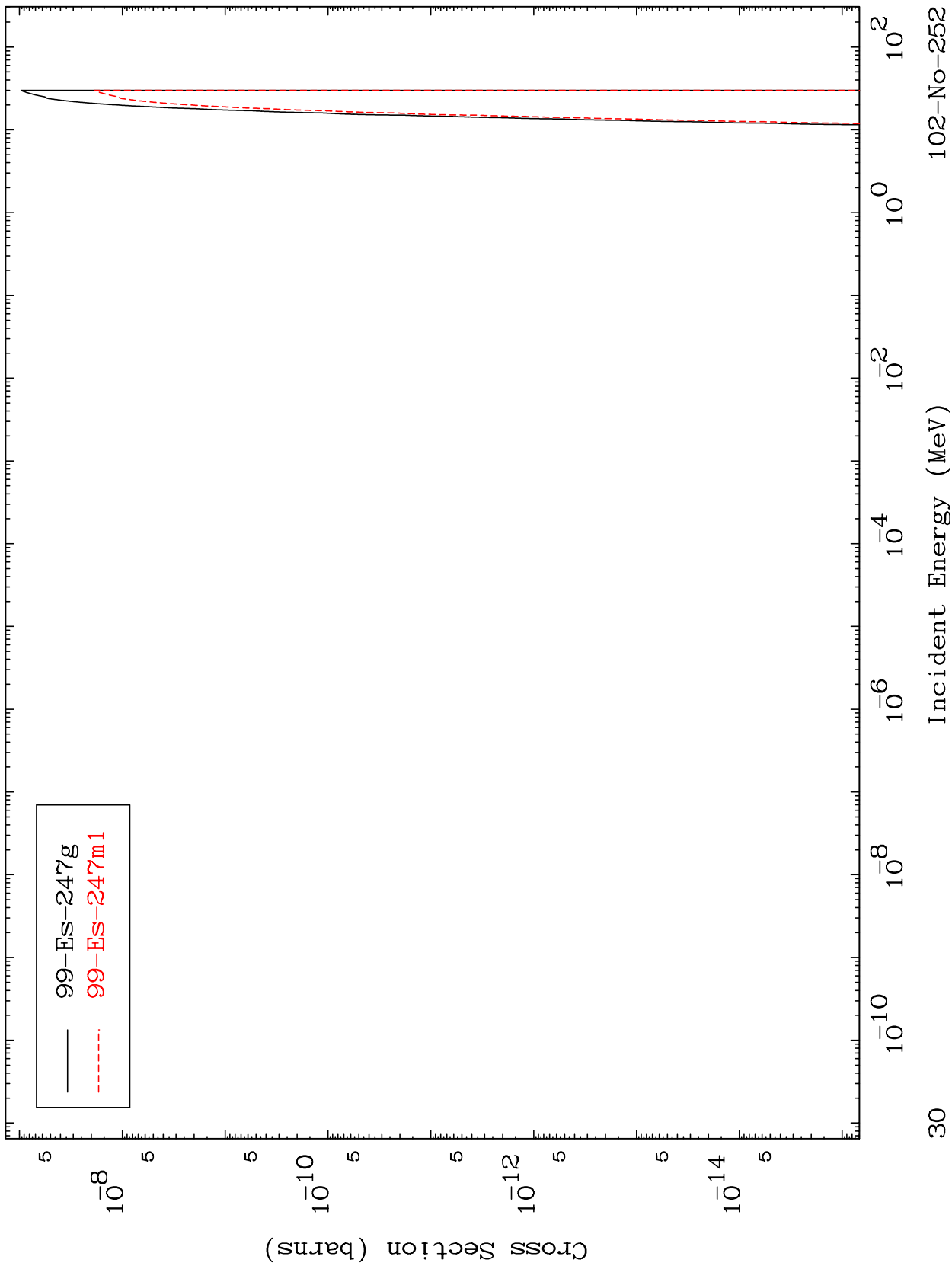
102-No-252

MAT 9970

(n,n') p α

102-No-252

Radionuclide Production Cross Section



— 99-Es-247g
- - - 99-Es-247m1

MAT 9970

(n,d) α

102-No-252

Radionuclide Production Cross Section

