

Program EVALPLOT  
(Version 2021-1)

by

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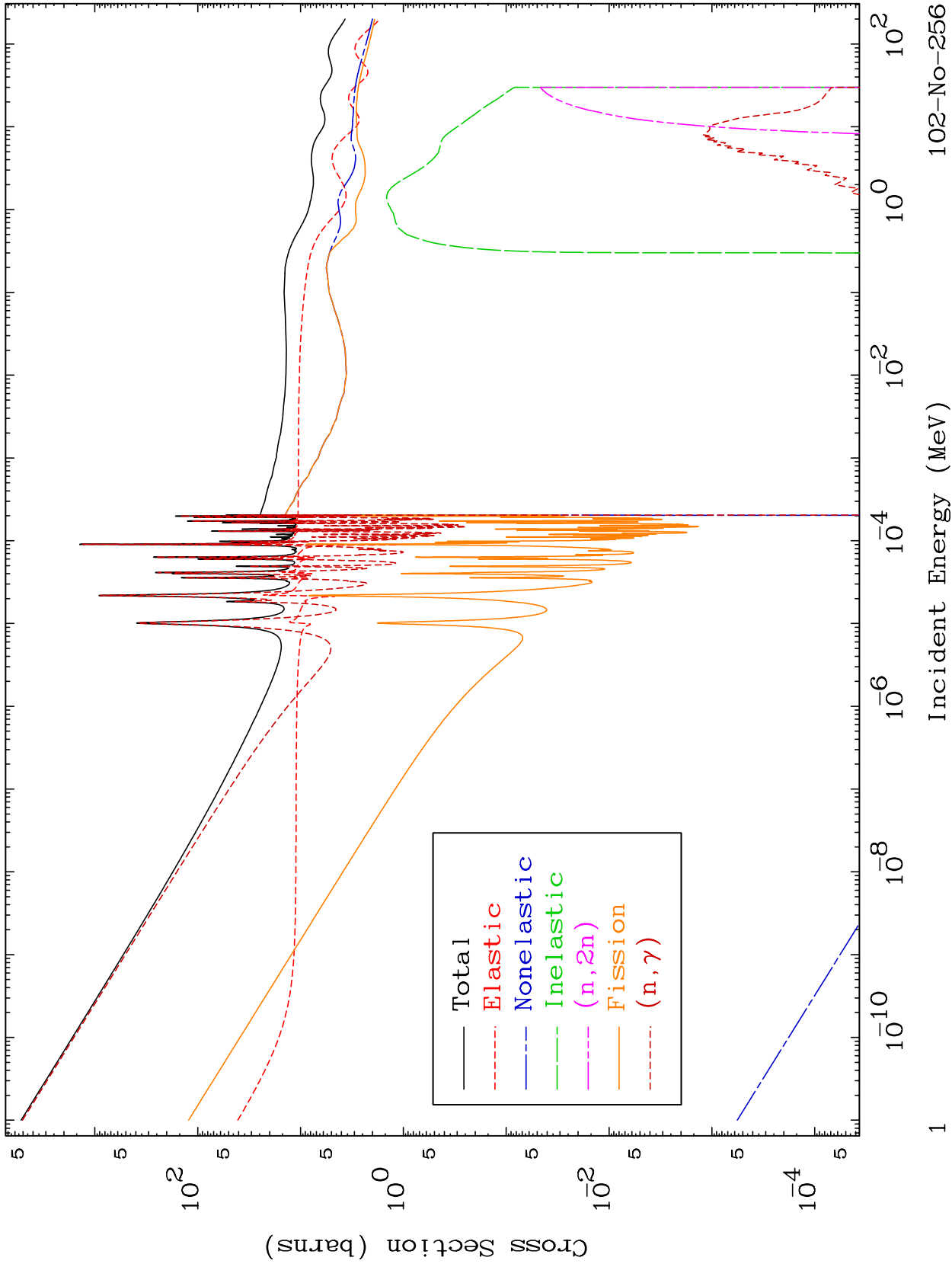
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9974

Neutron Major  
293 Kelvin Cross Sections

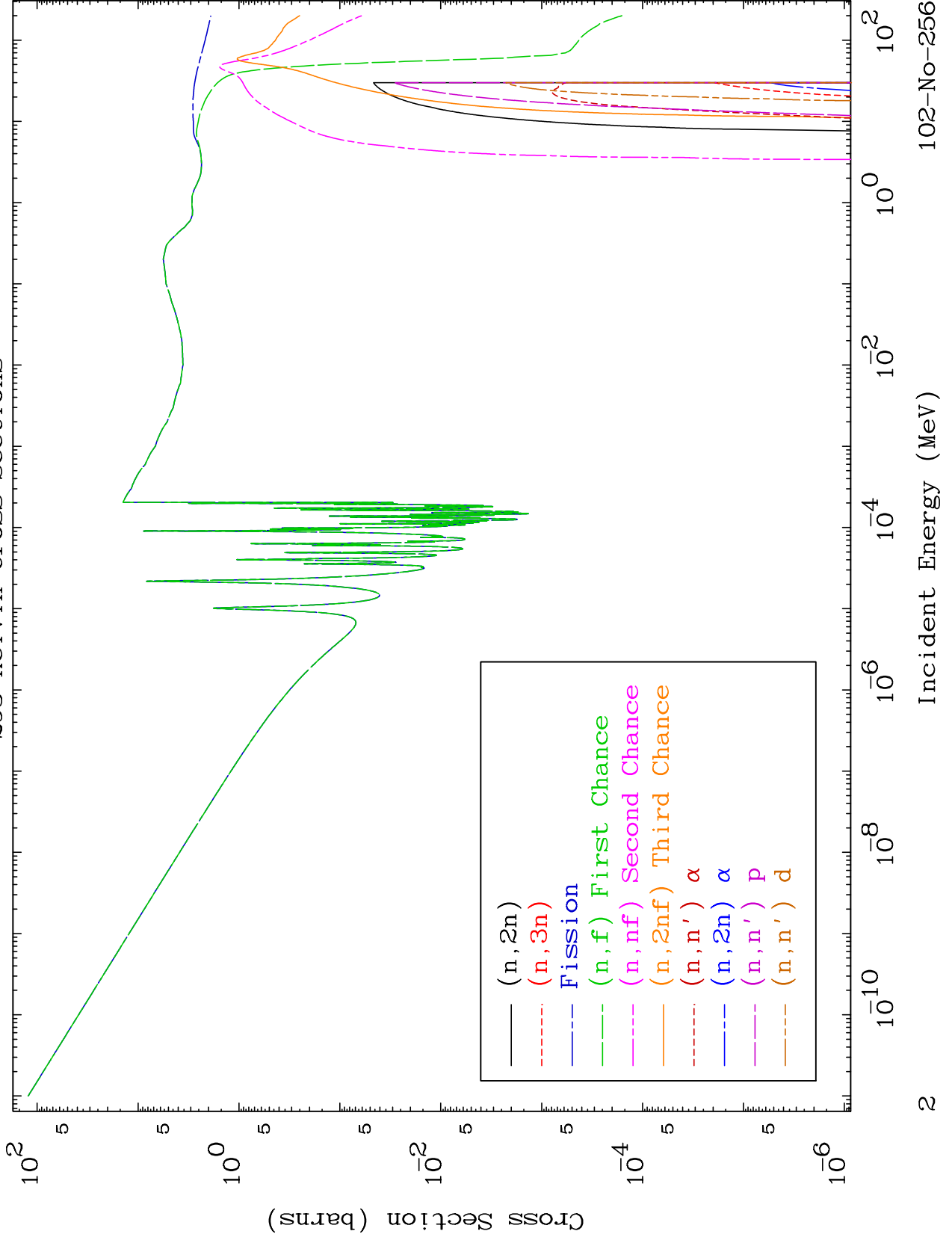
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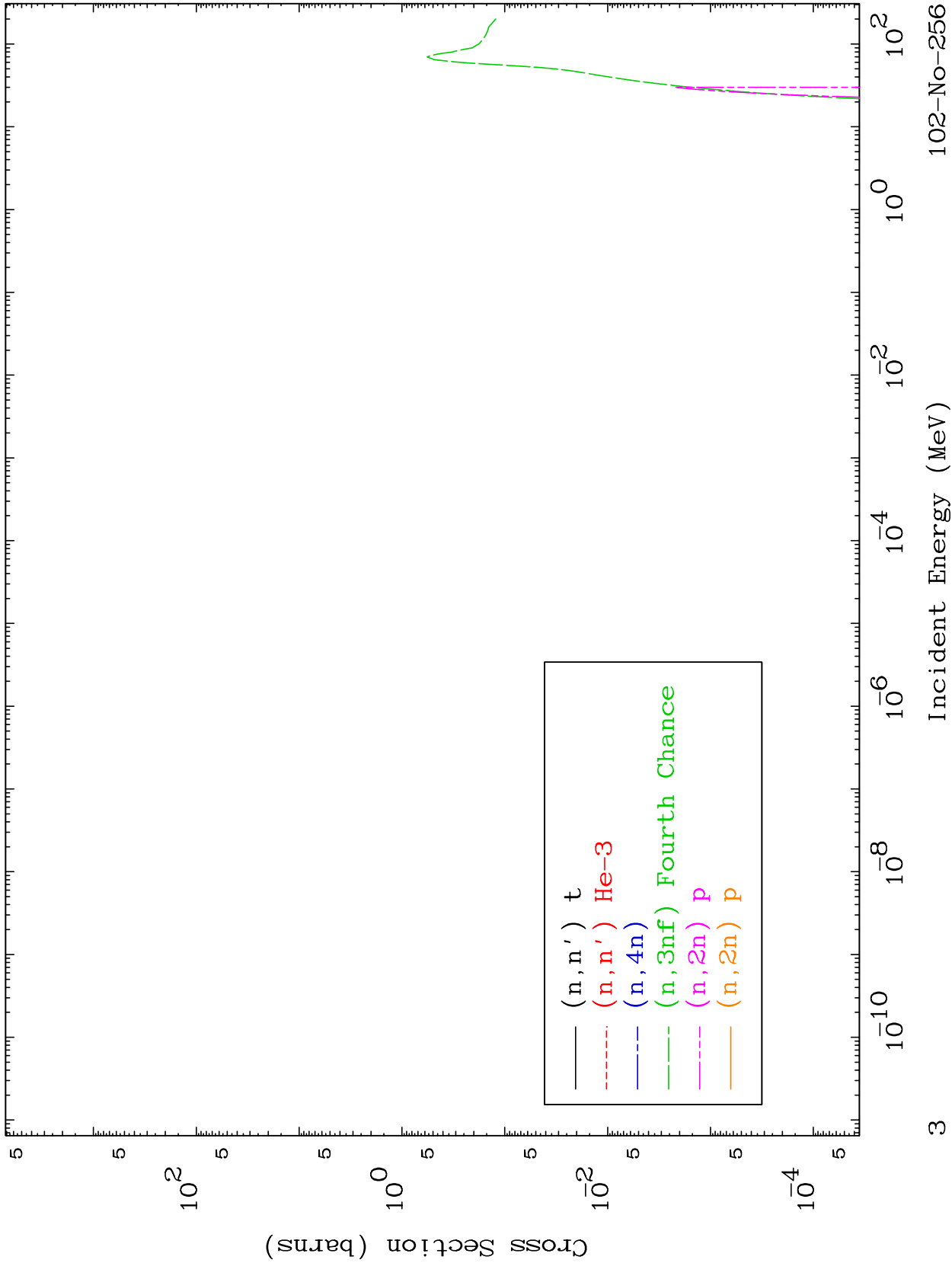


MAT 9974

Neutron Absorption  
293 Kelvin Cross Sections

102-No-256

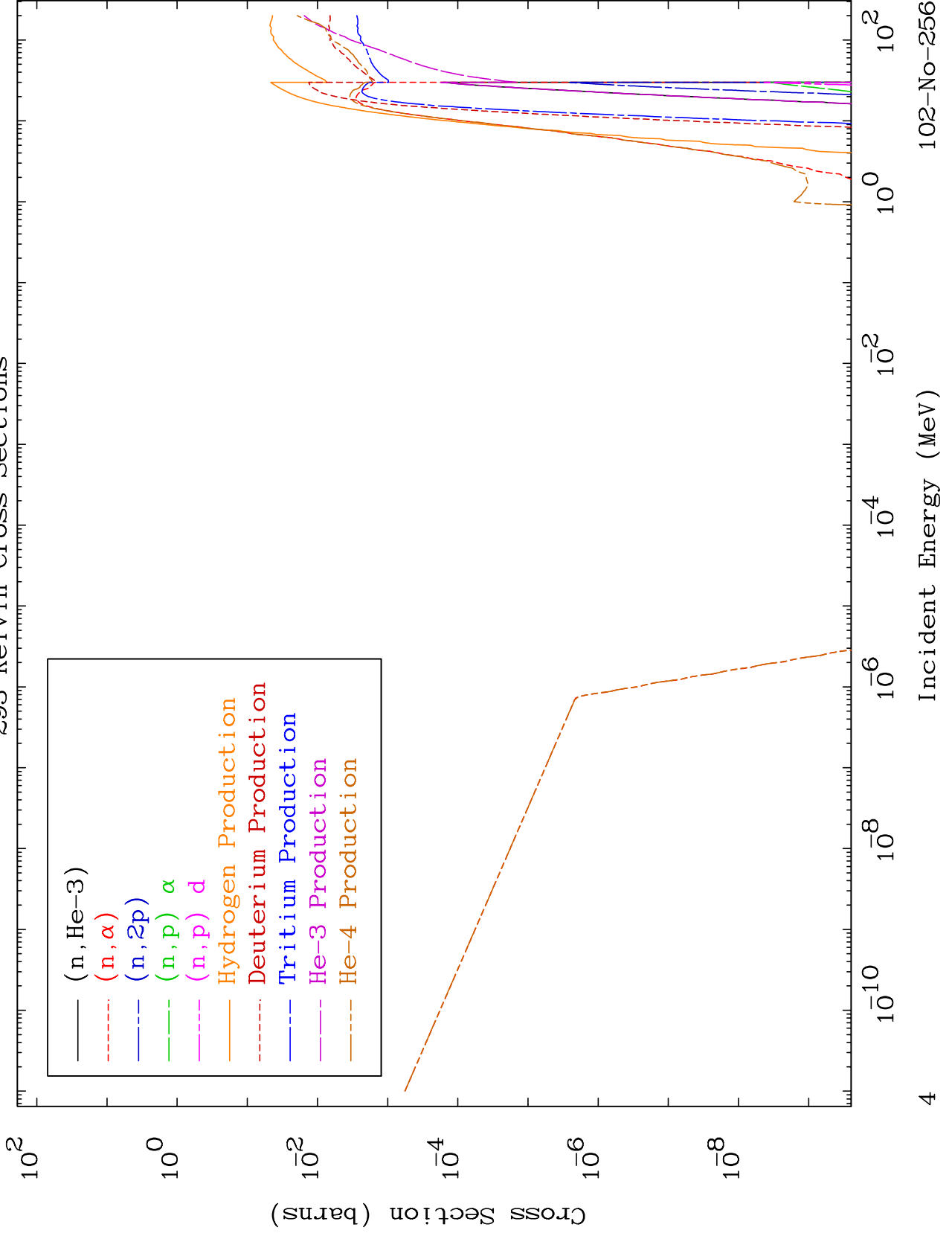




MAT 9974

Neutron Absorption  
293 Kelvin Cross Sections

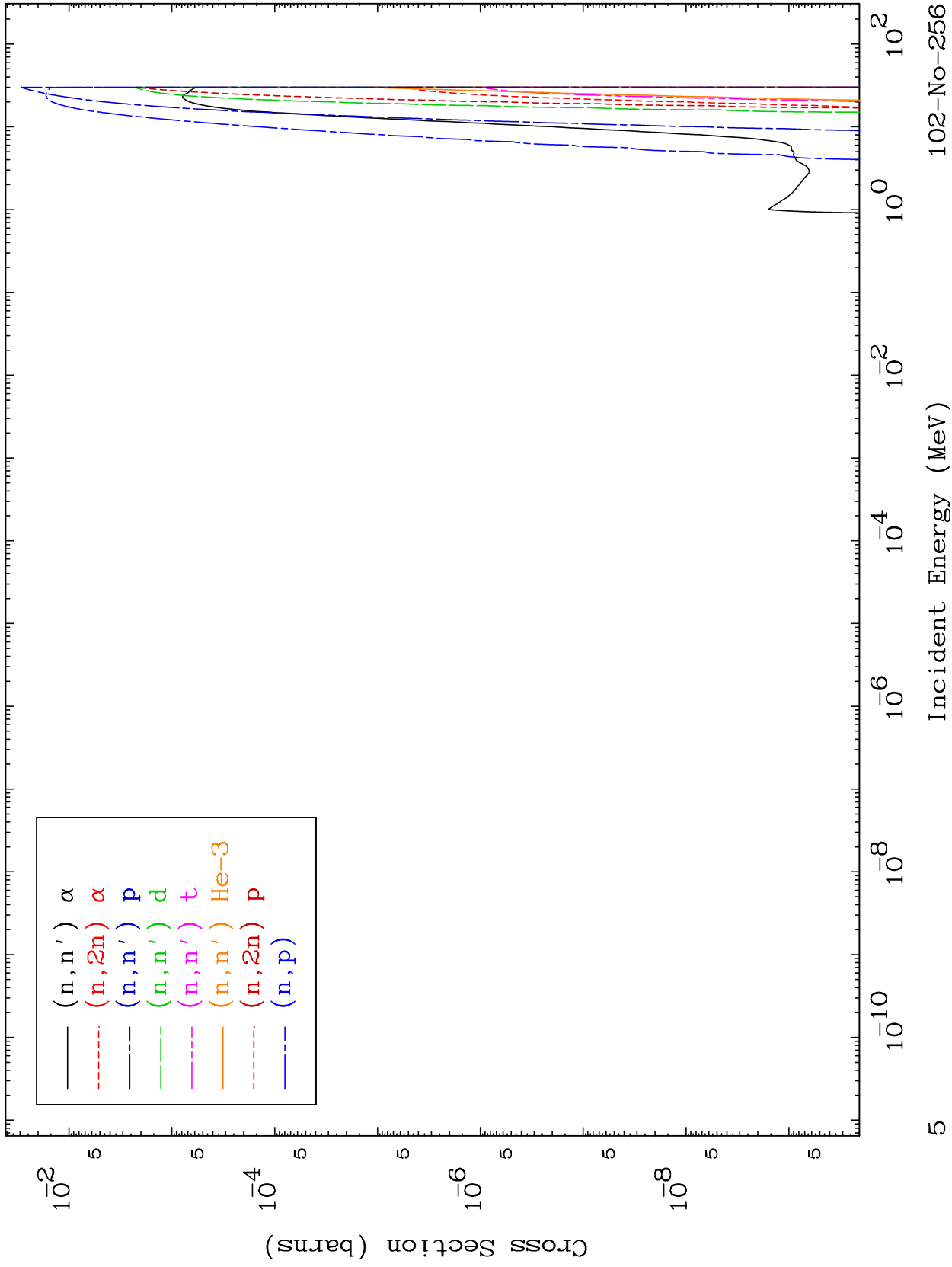
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MAT 9974

Charged Particle  
293 Kelvin Cross Sections

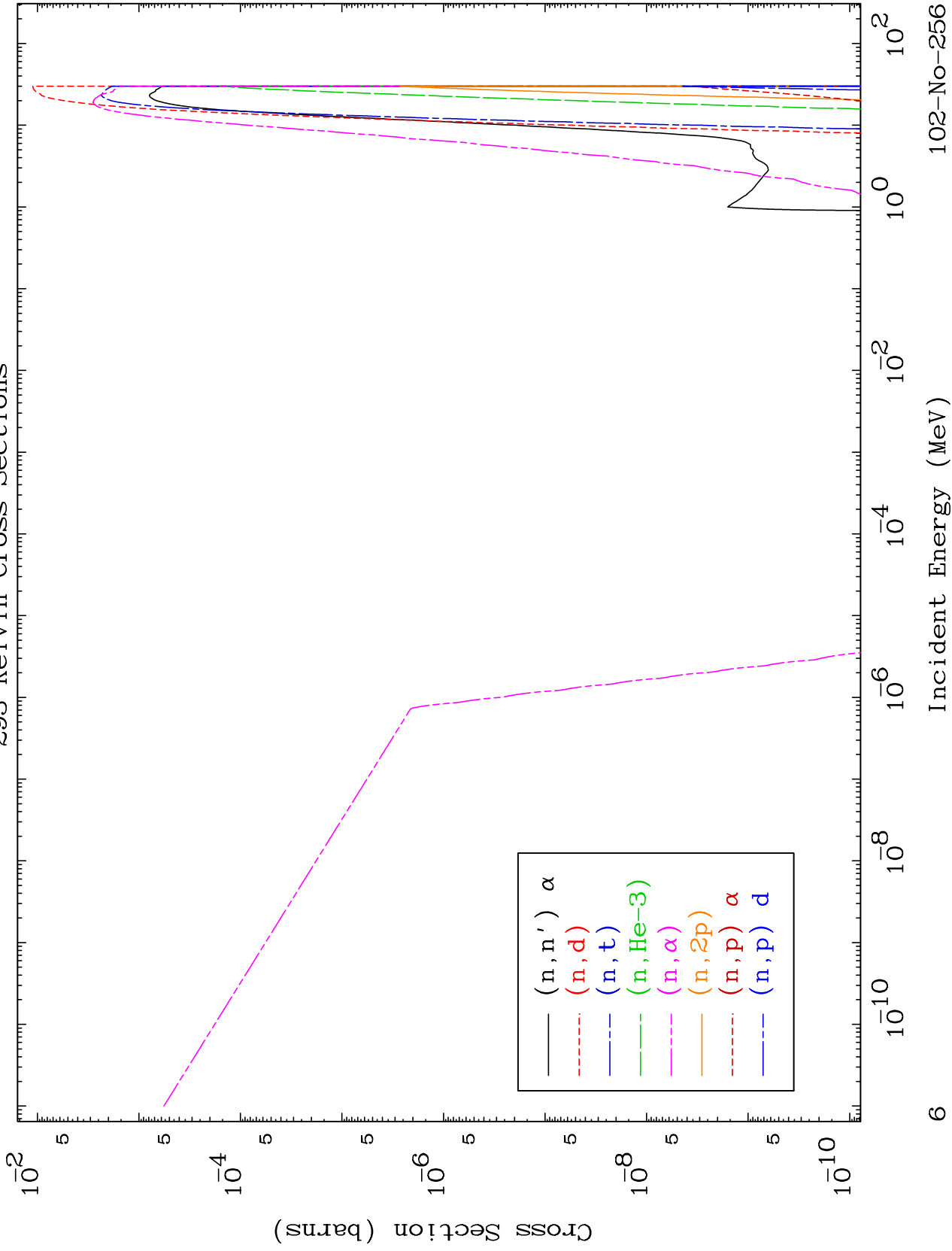
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MAT 9974

Charged Particle  
293 Kelvin Cross Sections

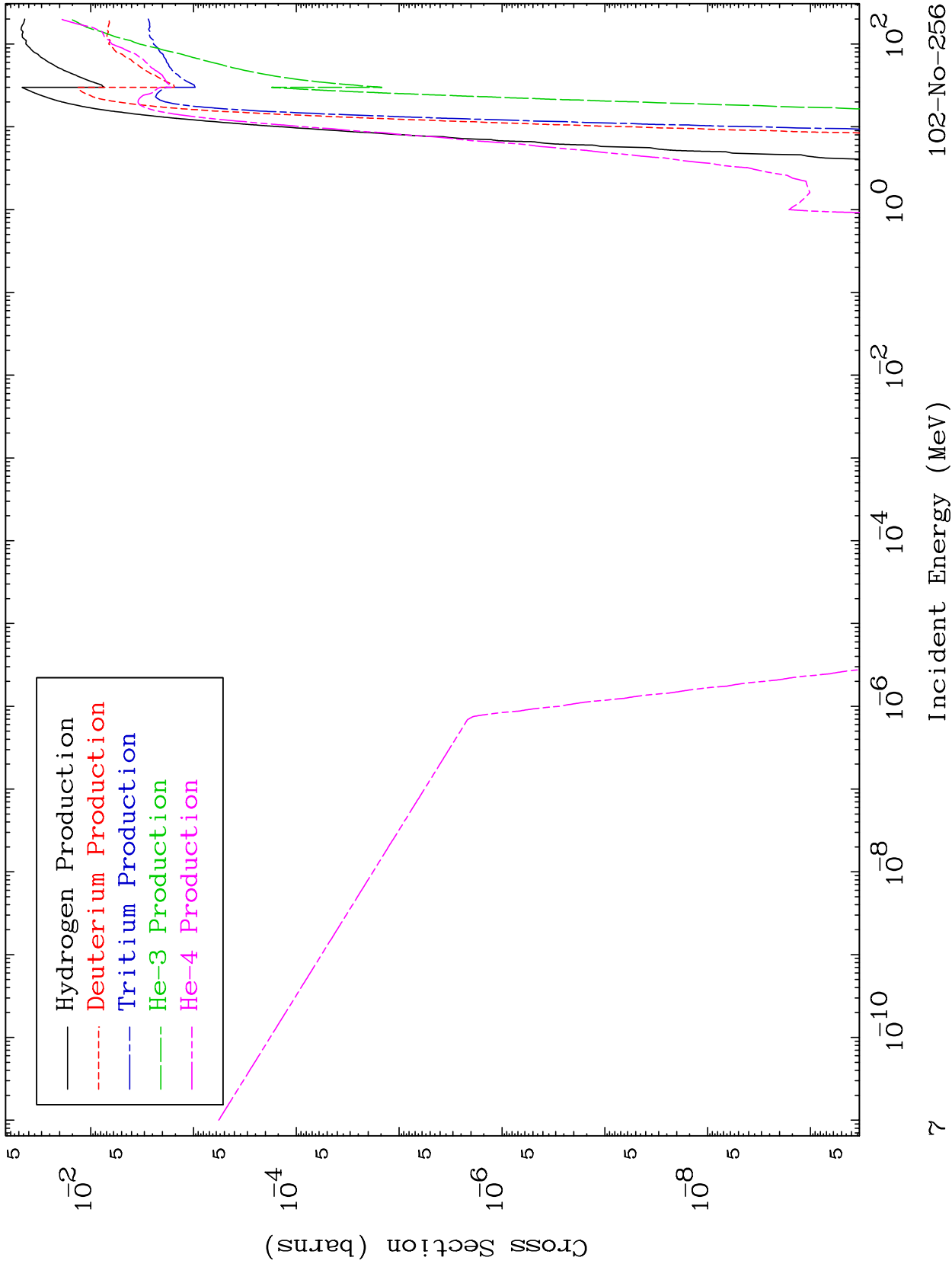
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Particle Production  
293 Kelvin Cross Sections

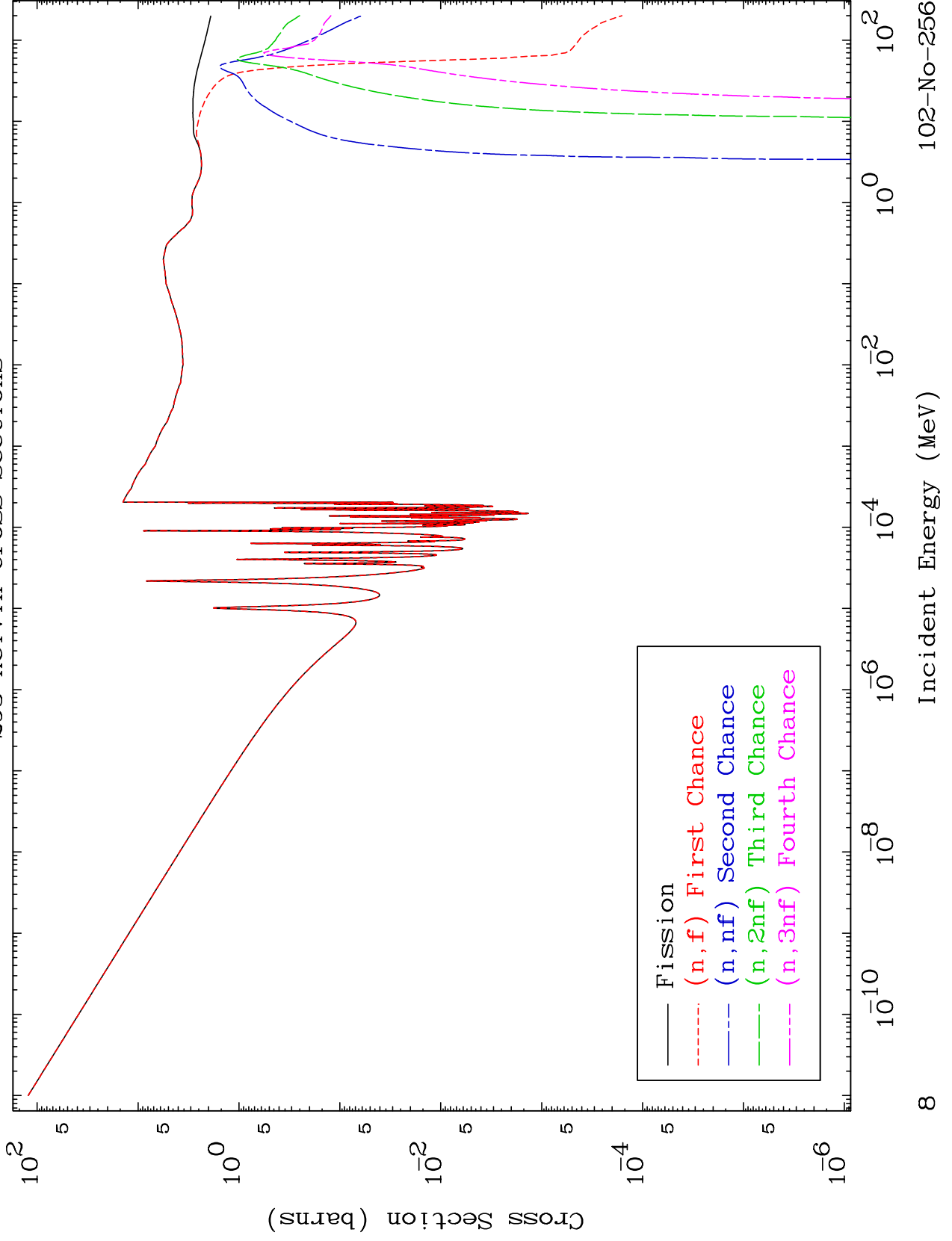
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Fission  
293 Kelvin Cross Sections

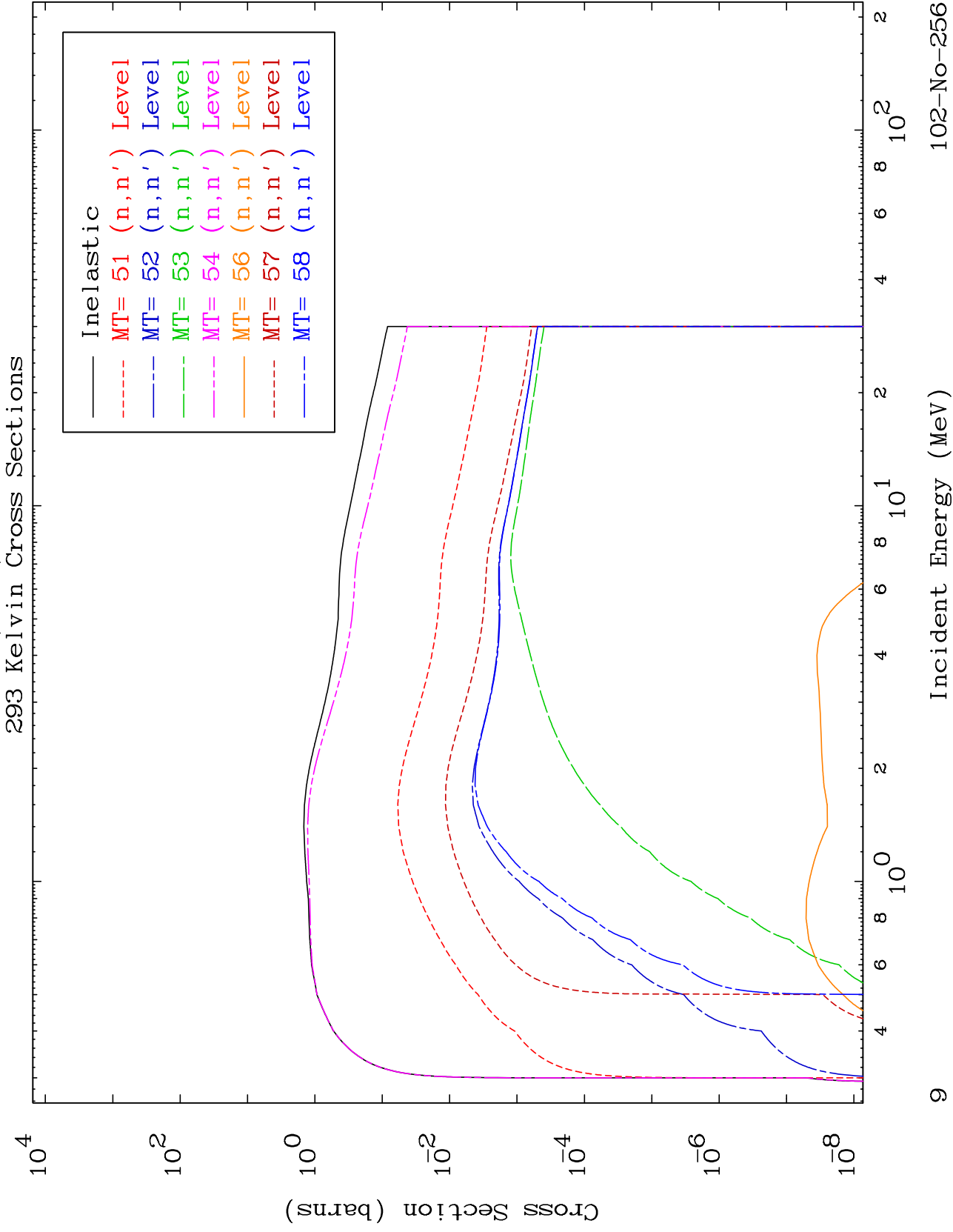
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(n,n') Levels

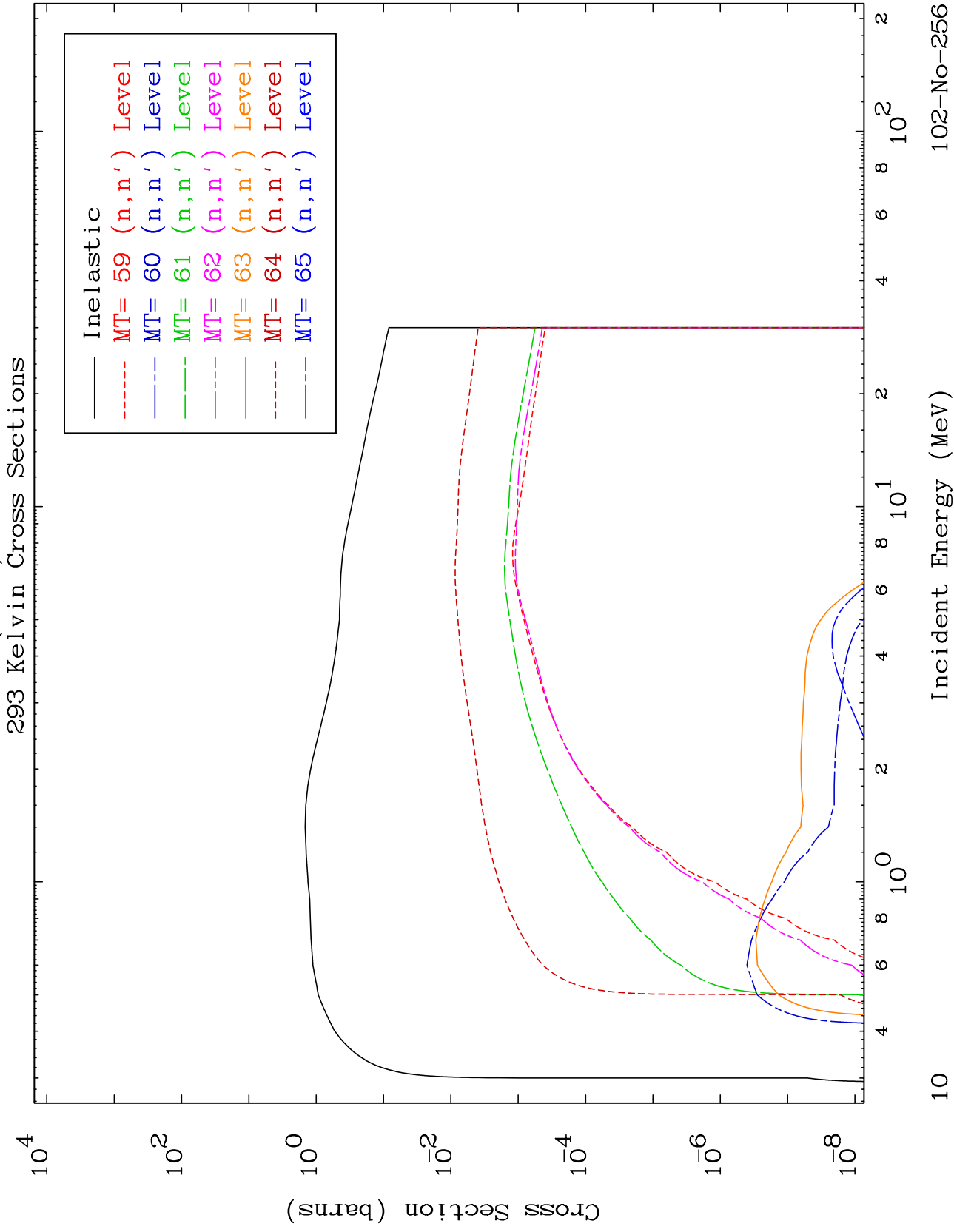
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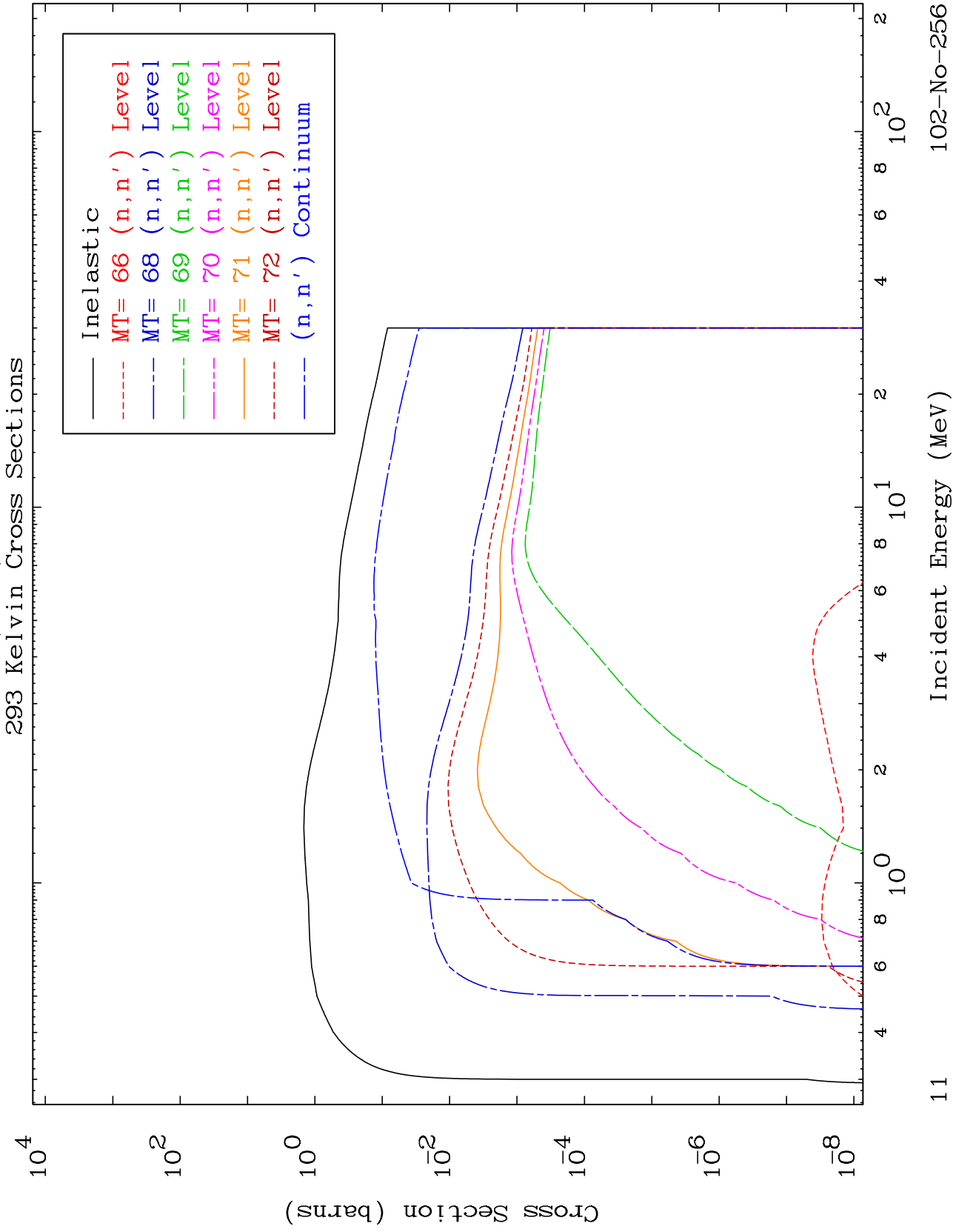


MAT 9974

(n,n') Levels

102-No-256

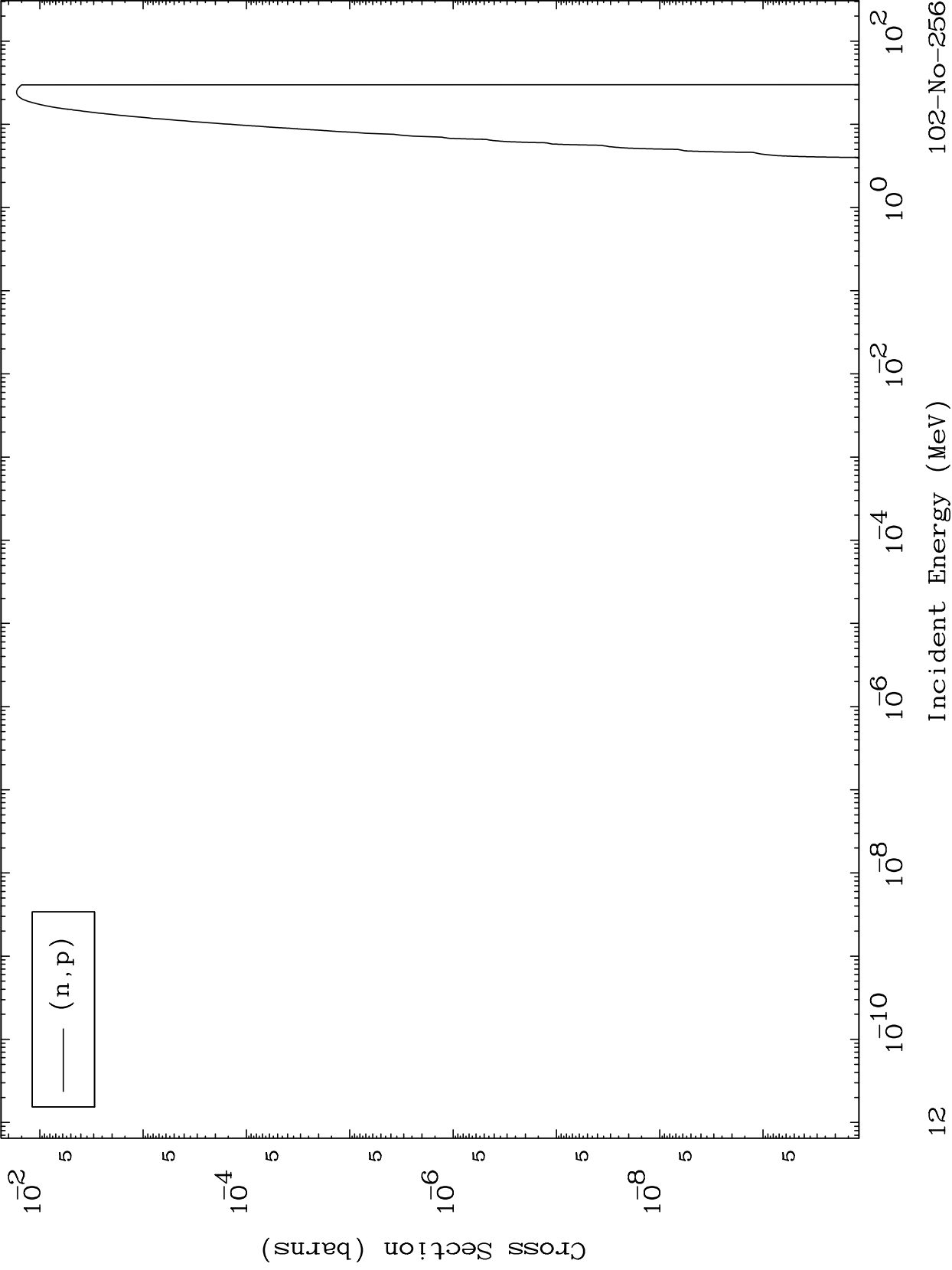




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(n,p) Levels  
293 Kelvin Cross Sections

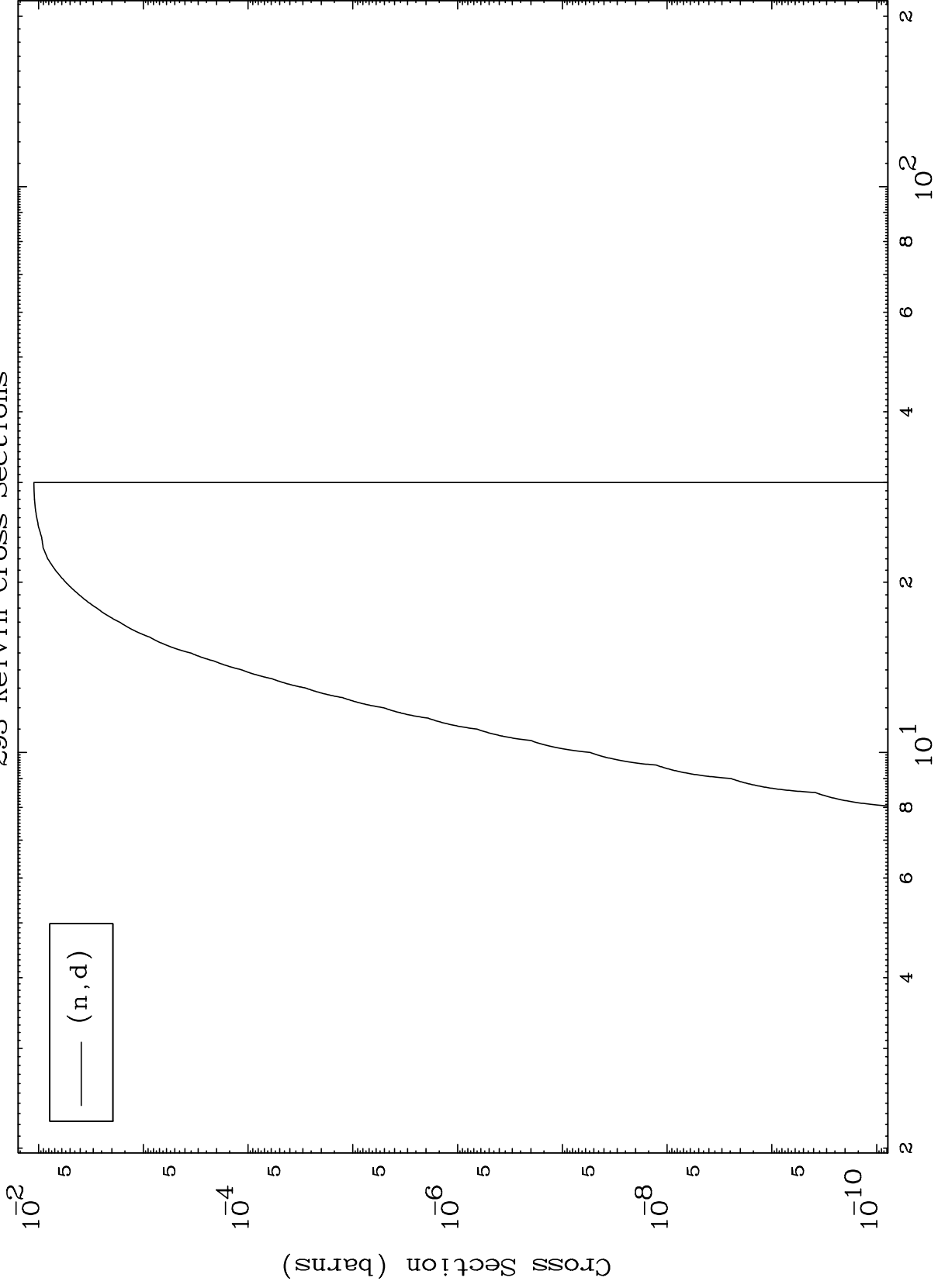
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MAT 9974

(n,d) Levels  
293 Kelvin Cross Sections

102-No-256



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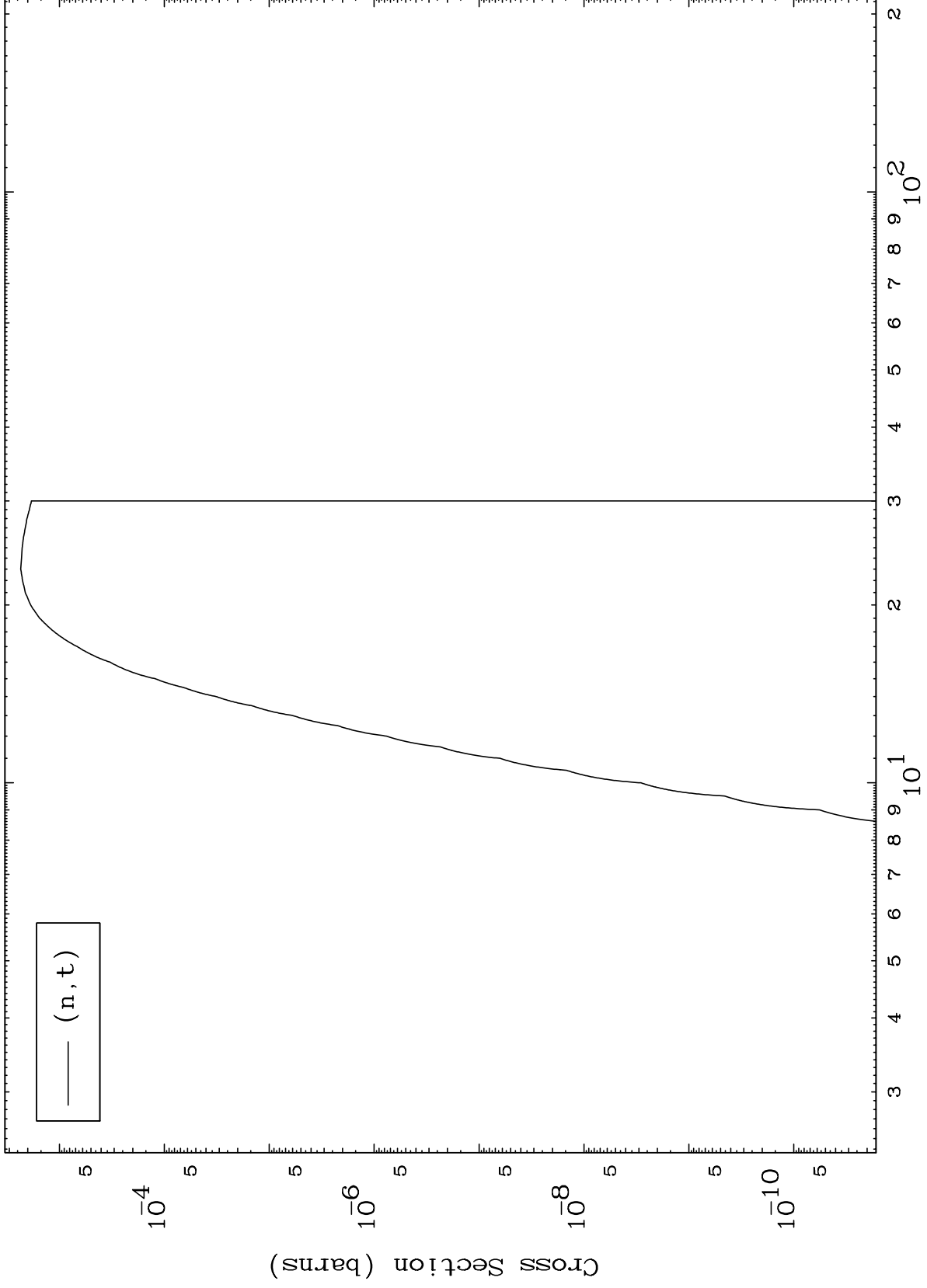
Incident Energy (MeV)

102-No-256

MAT 9974

(n,t) Levels  
293 Kelvin Cross Sections

102-No-256



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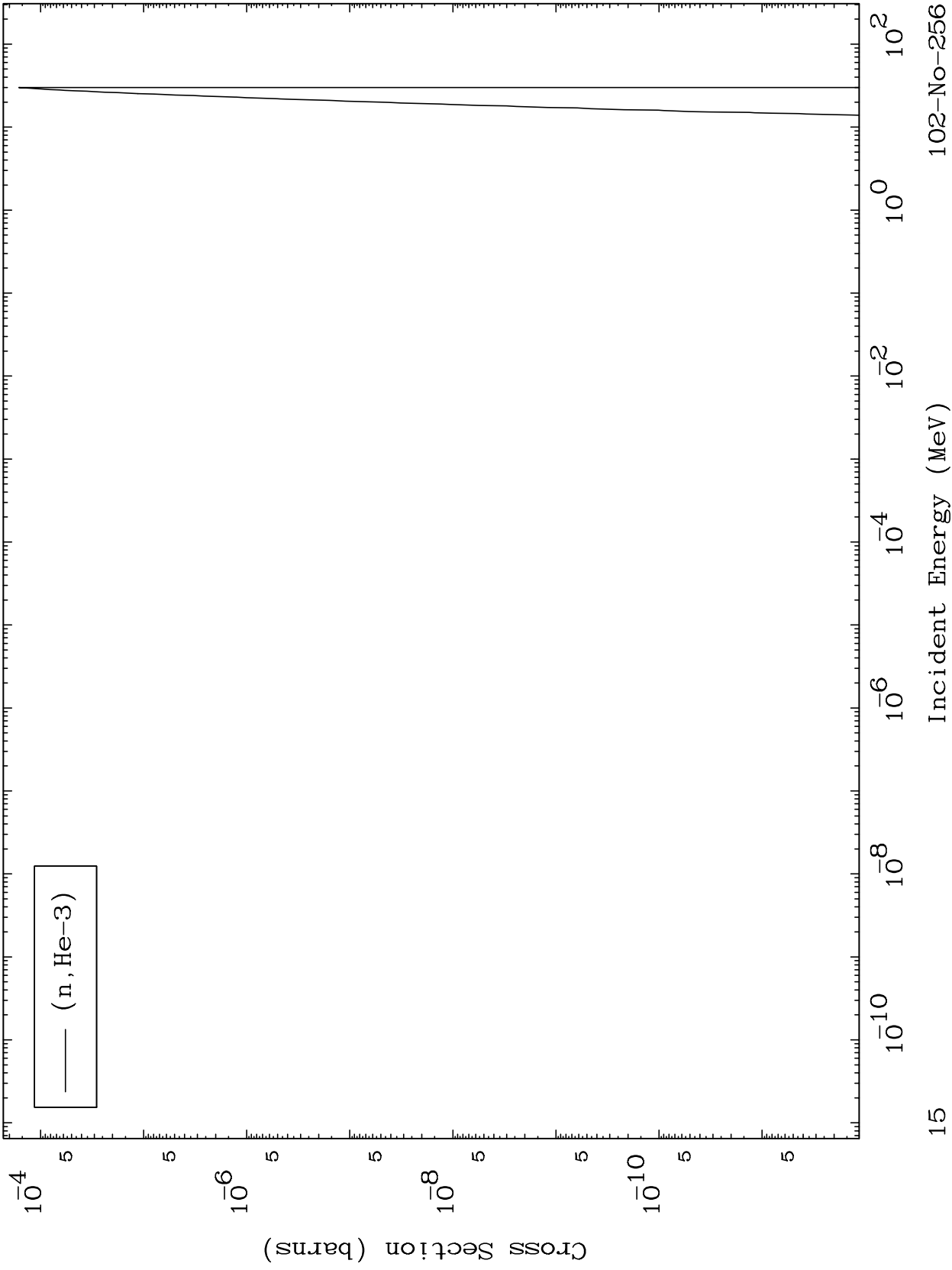
Incident Energy (MeV)

102-No-256

MAT 9974

(n,He3) Levels  
293 Kelvin Cross Sections

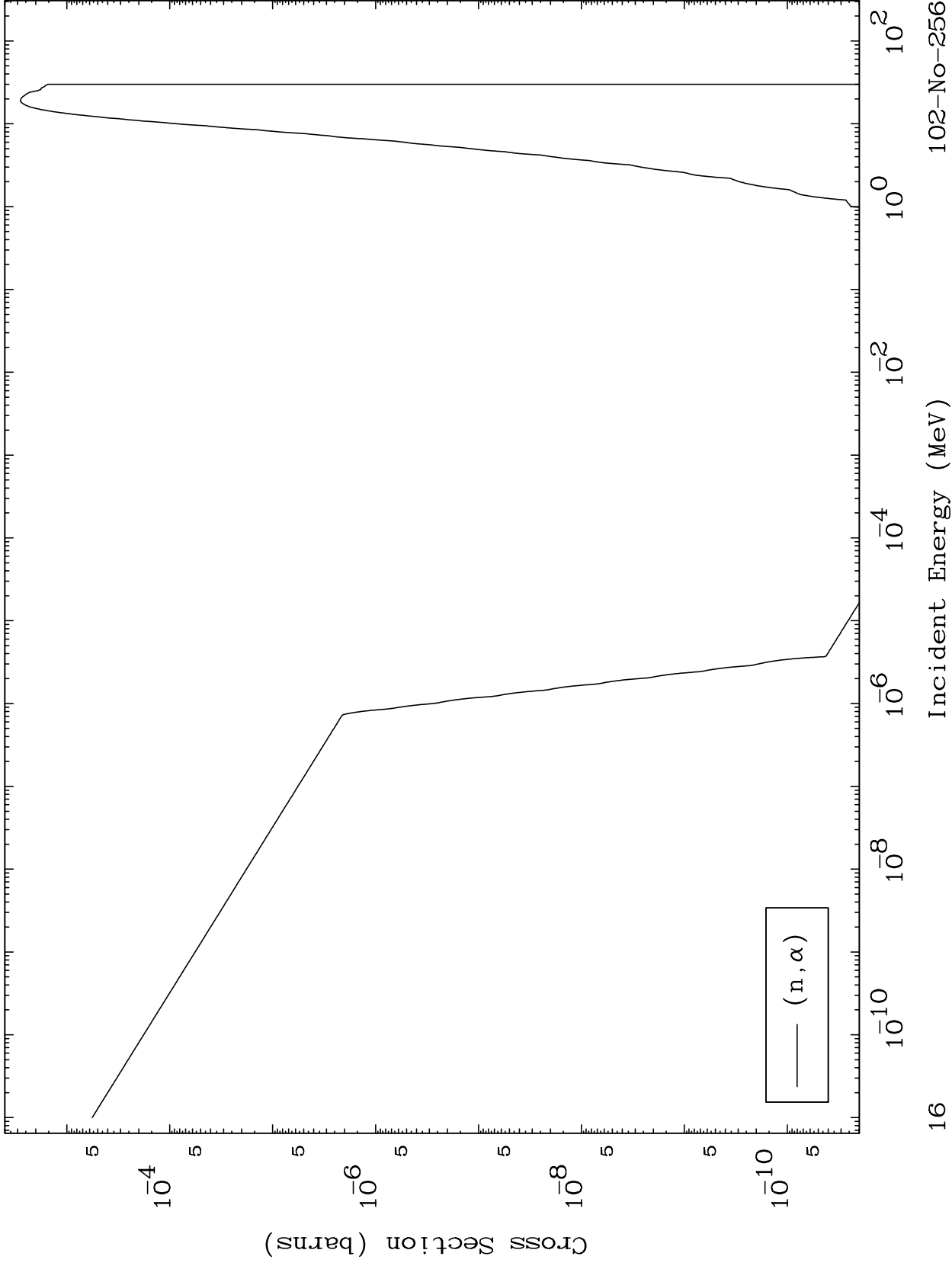
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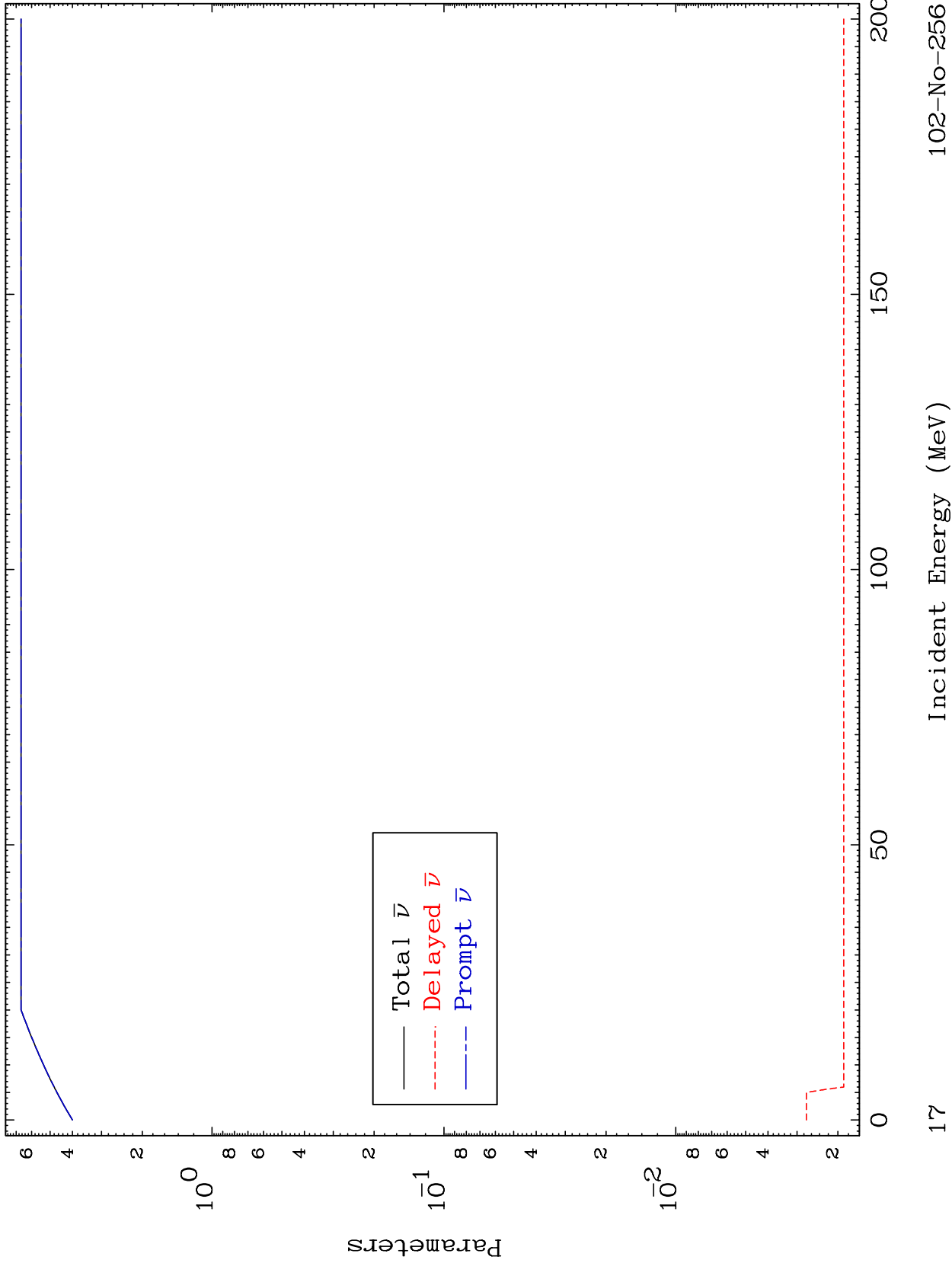


MAT 9974

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

102-No-256

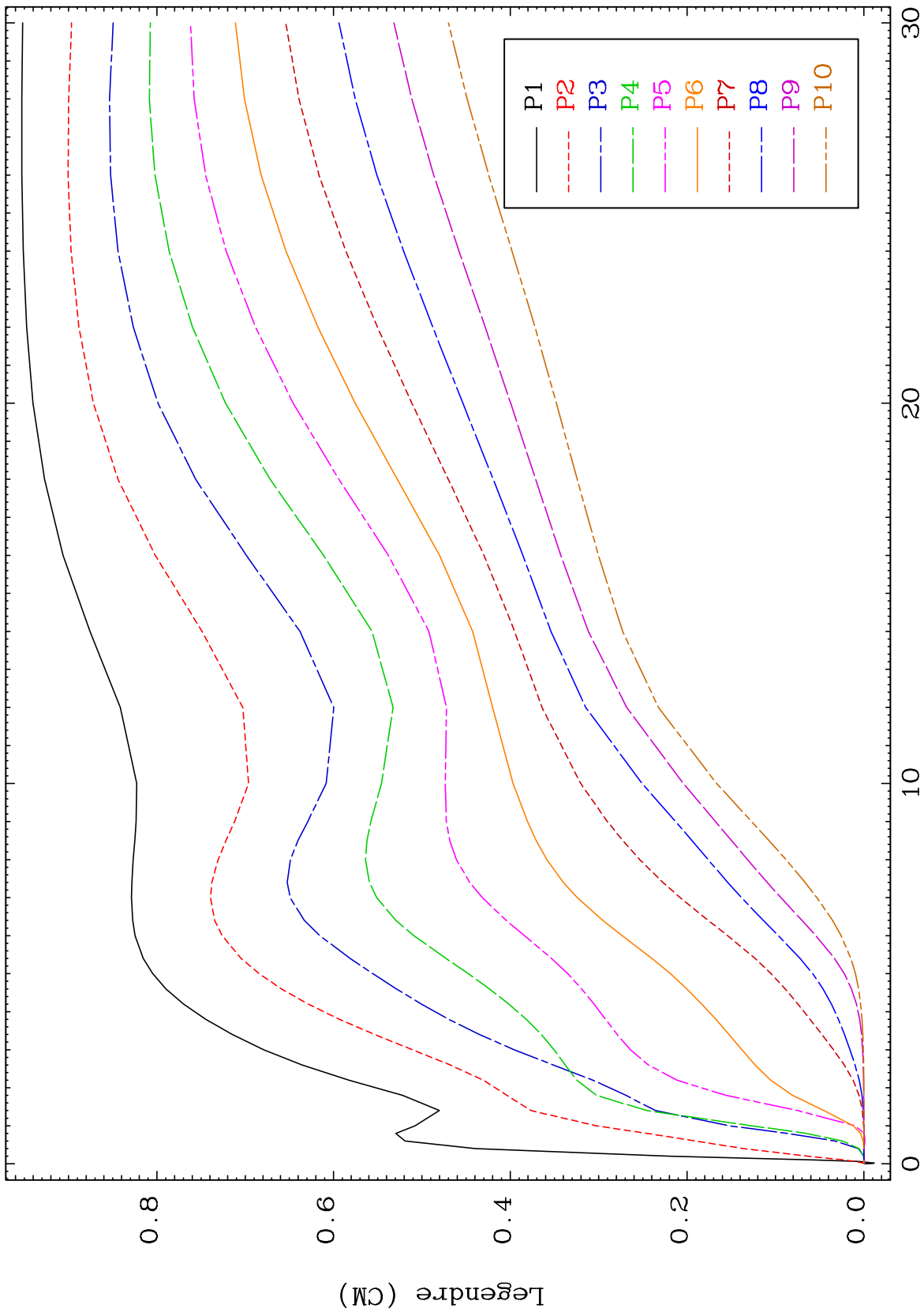




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Elastic Legendre Coefficients

102-No-256



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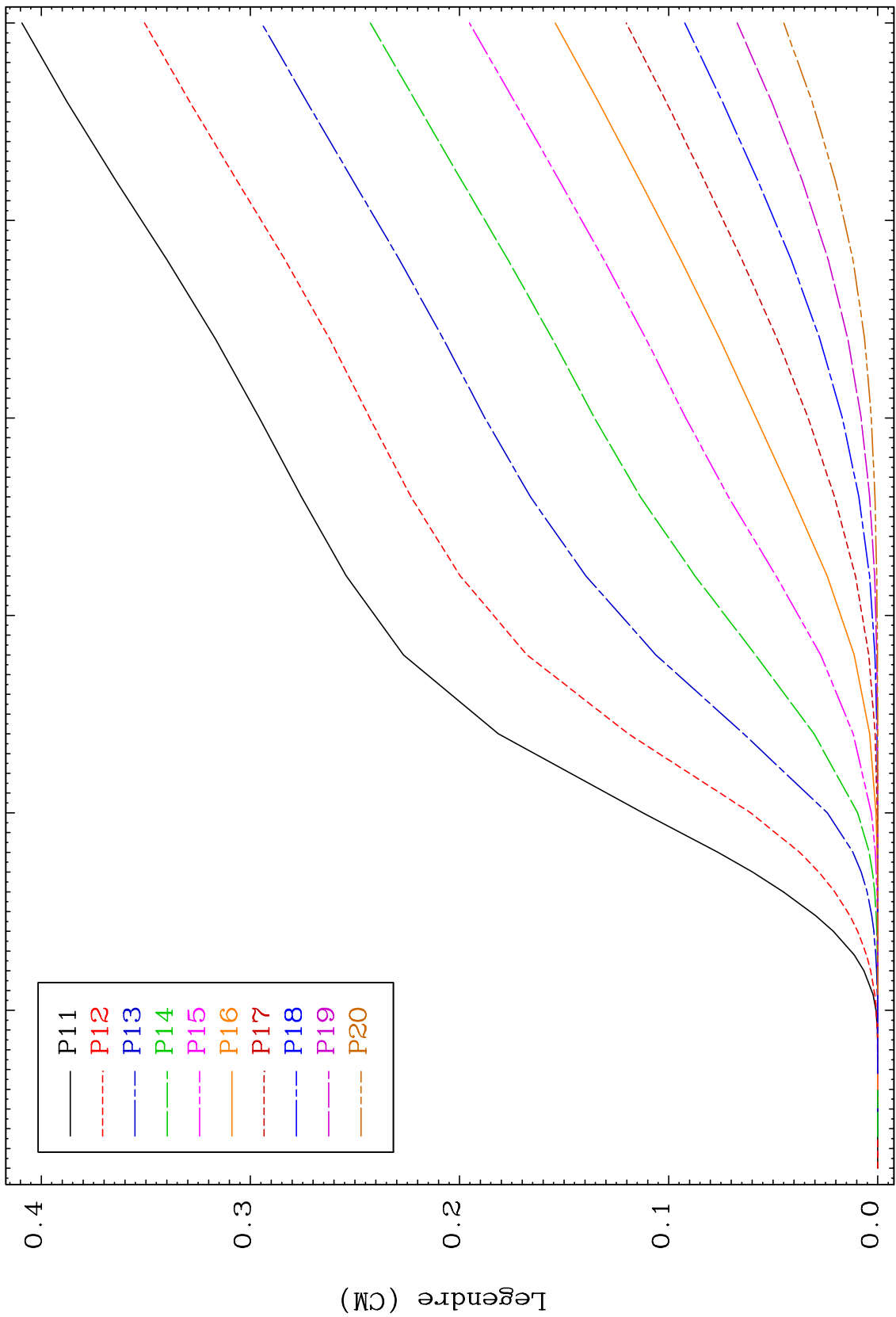
Incident Energy (MeV)

102-No-256

MAT 9974

Elastic Legendre Coefficients

102-No-256



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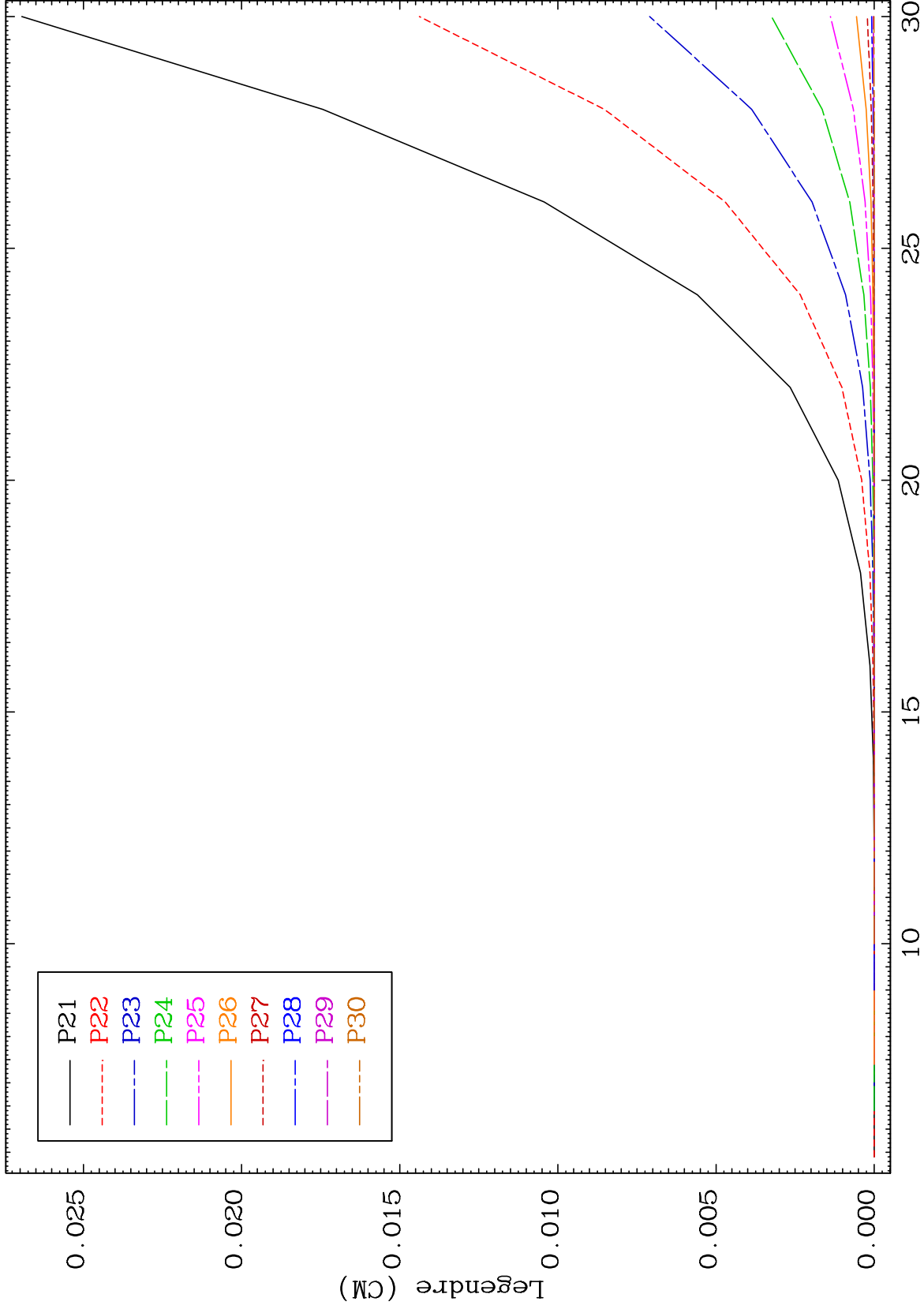
Incident Energy (MeV)

102-No-256

MAT 9974

### Elastic Legendre Coefficients

102-No-256



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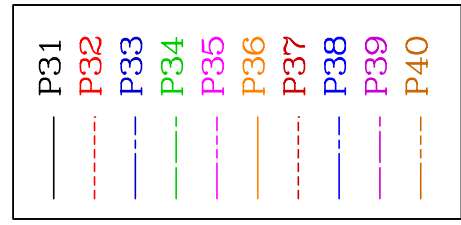
Incident Energy (MeV)

102-No-256

MAT 9974

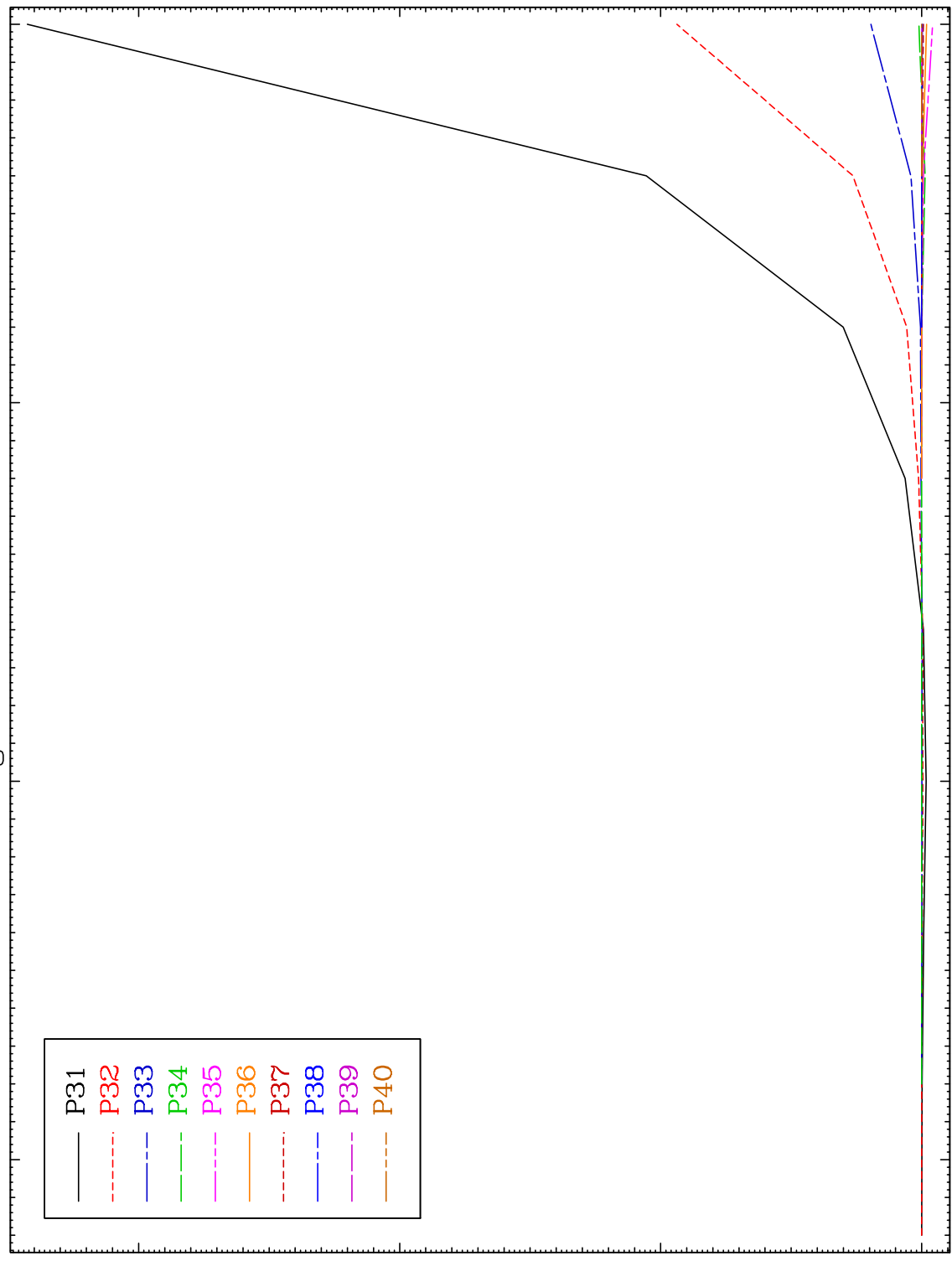
Elastic Legendre Coefficients

102-No-256



$\times 10^{-6}$

Legendre (CM)

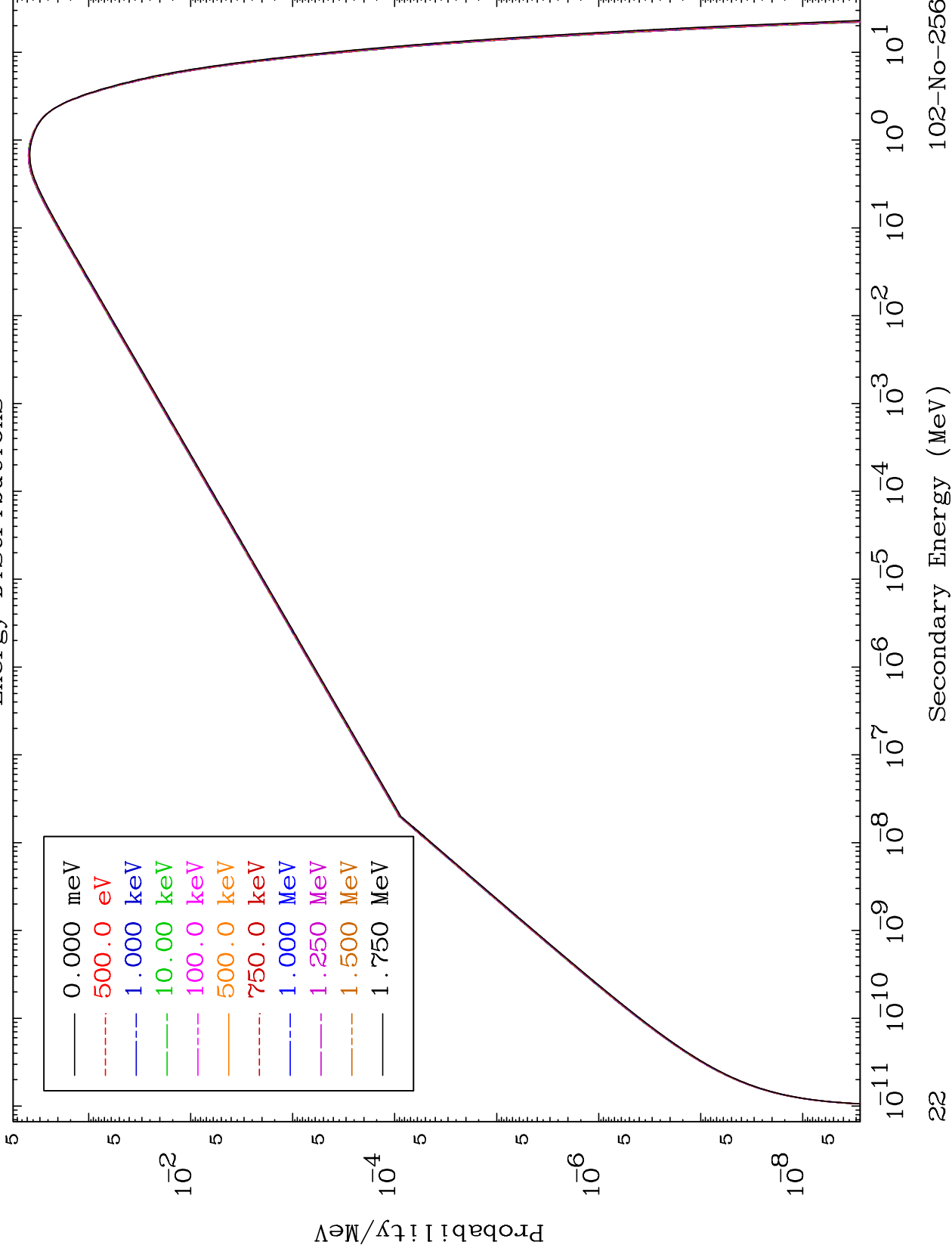


21

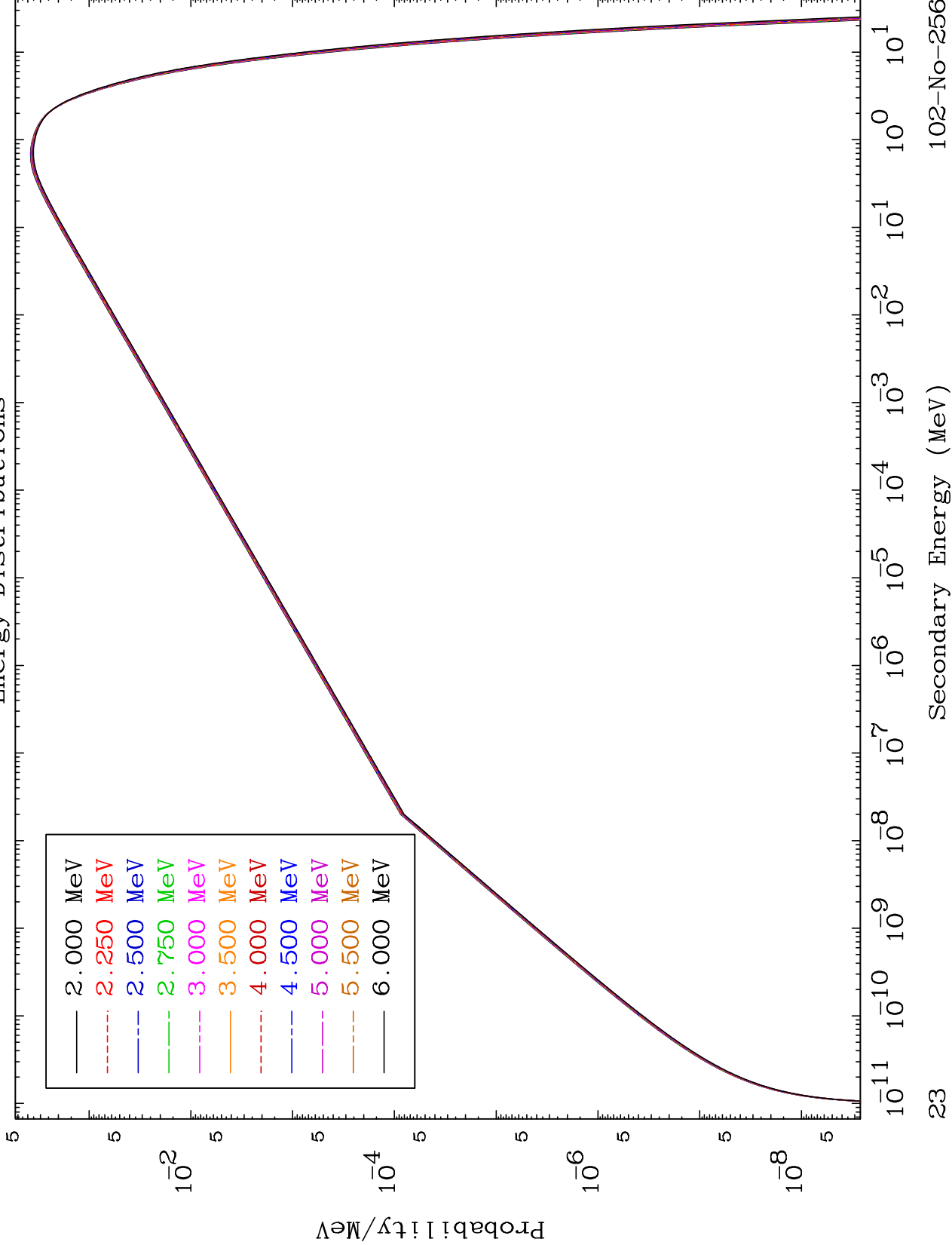
Incident Energy (MeV)

102-No-256

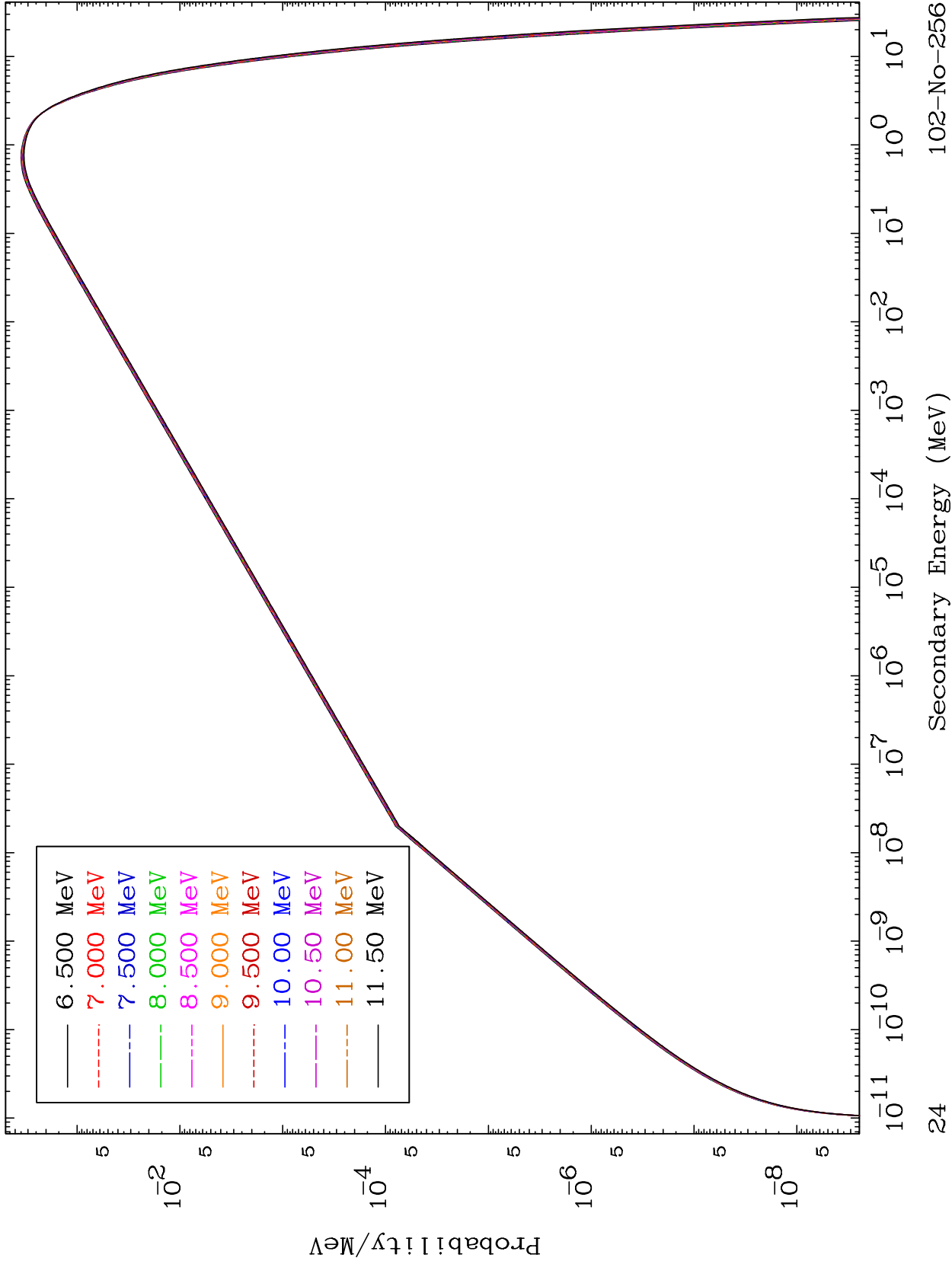
Fission Energy Distributions

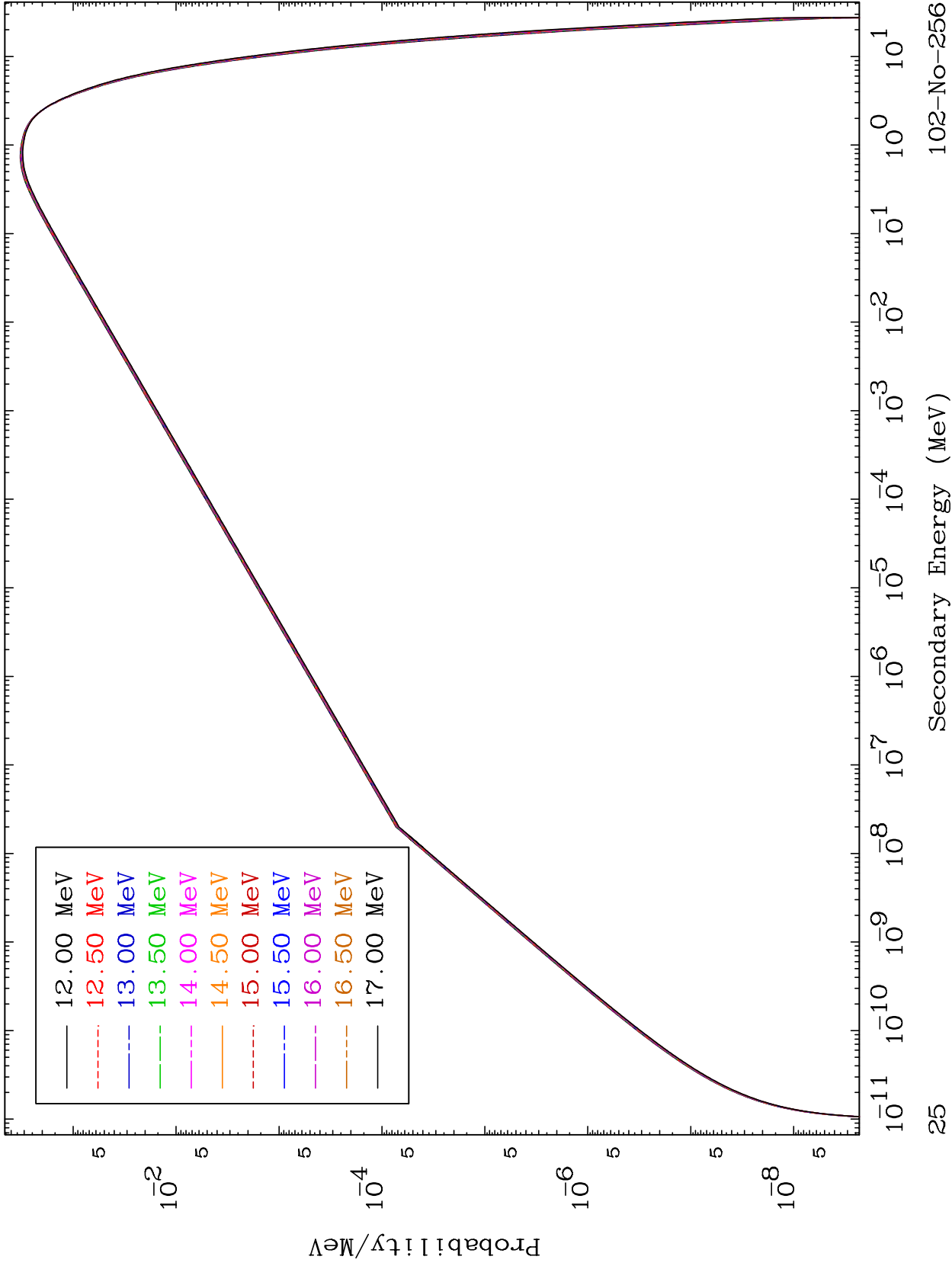


Fission Energy Distributions

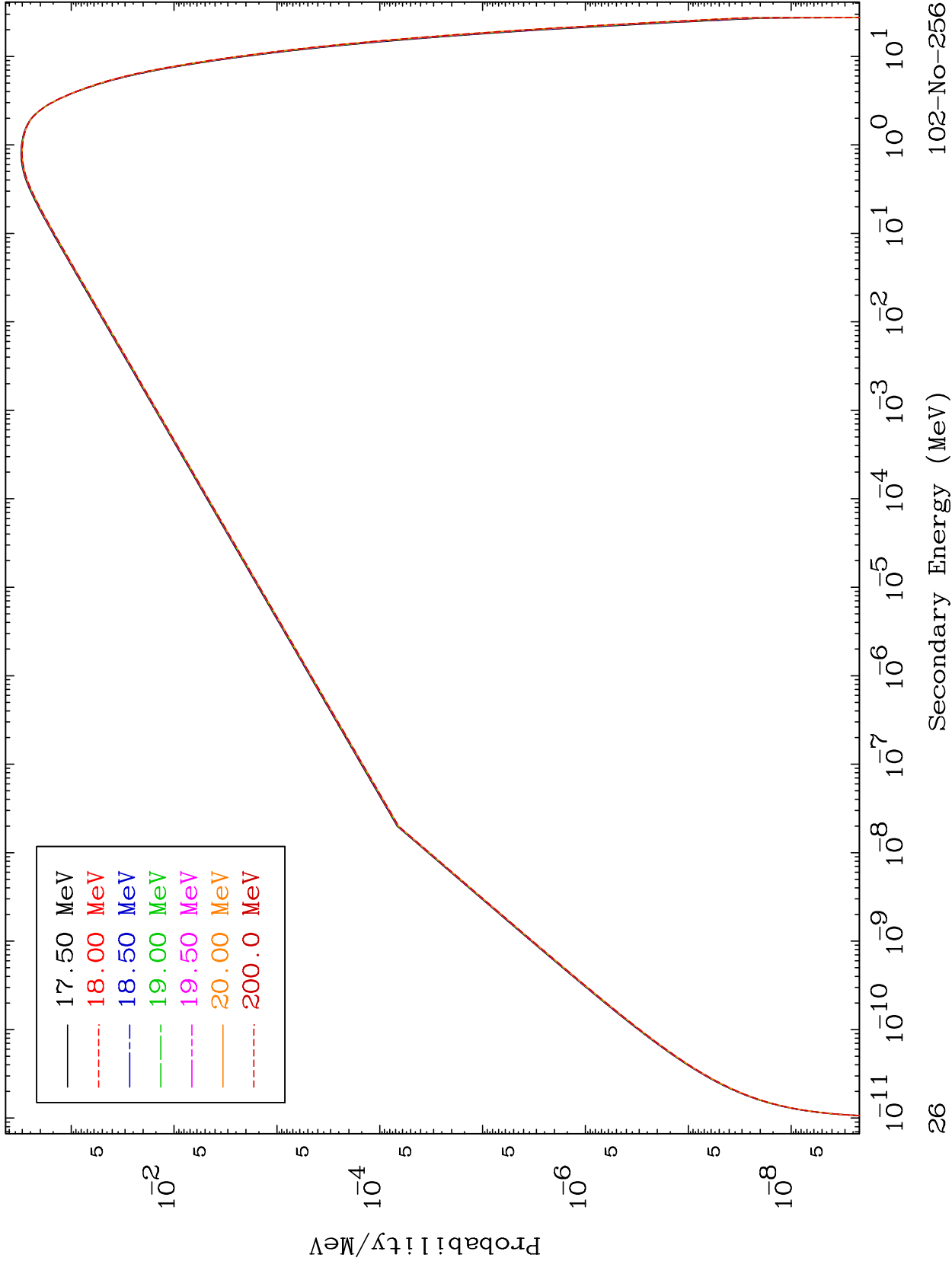


Fission Energy Distributions

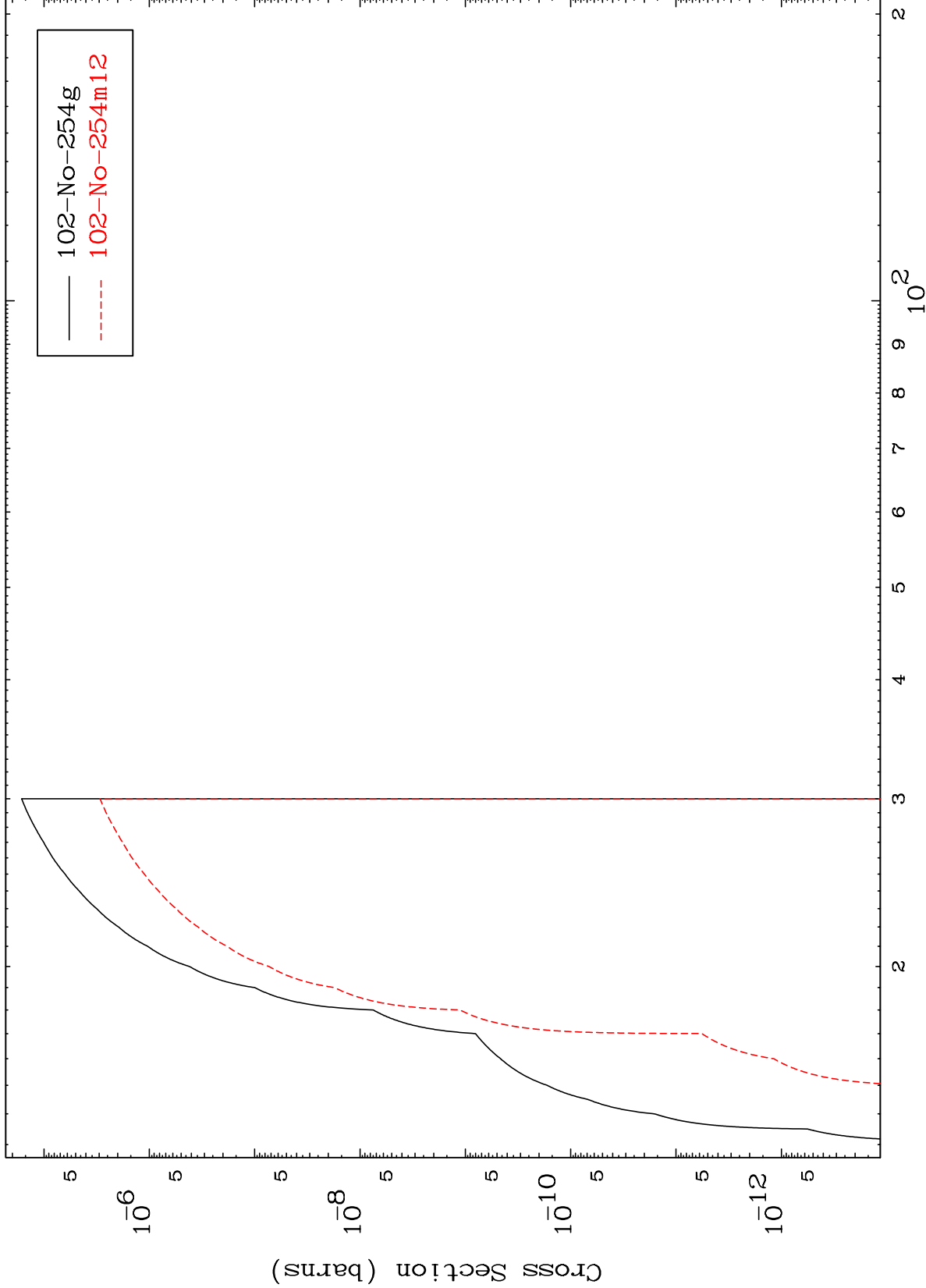




Fission Energy Distributions

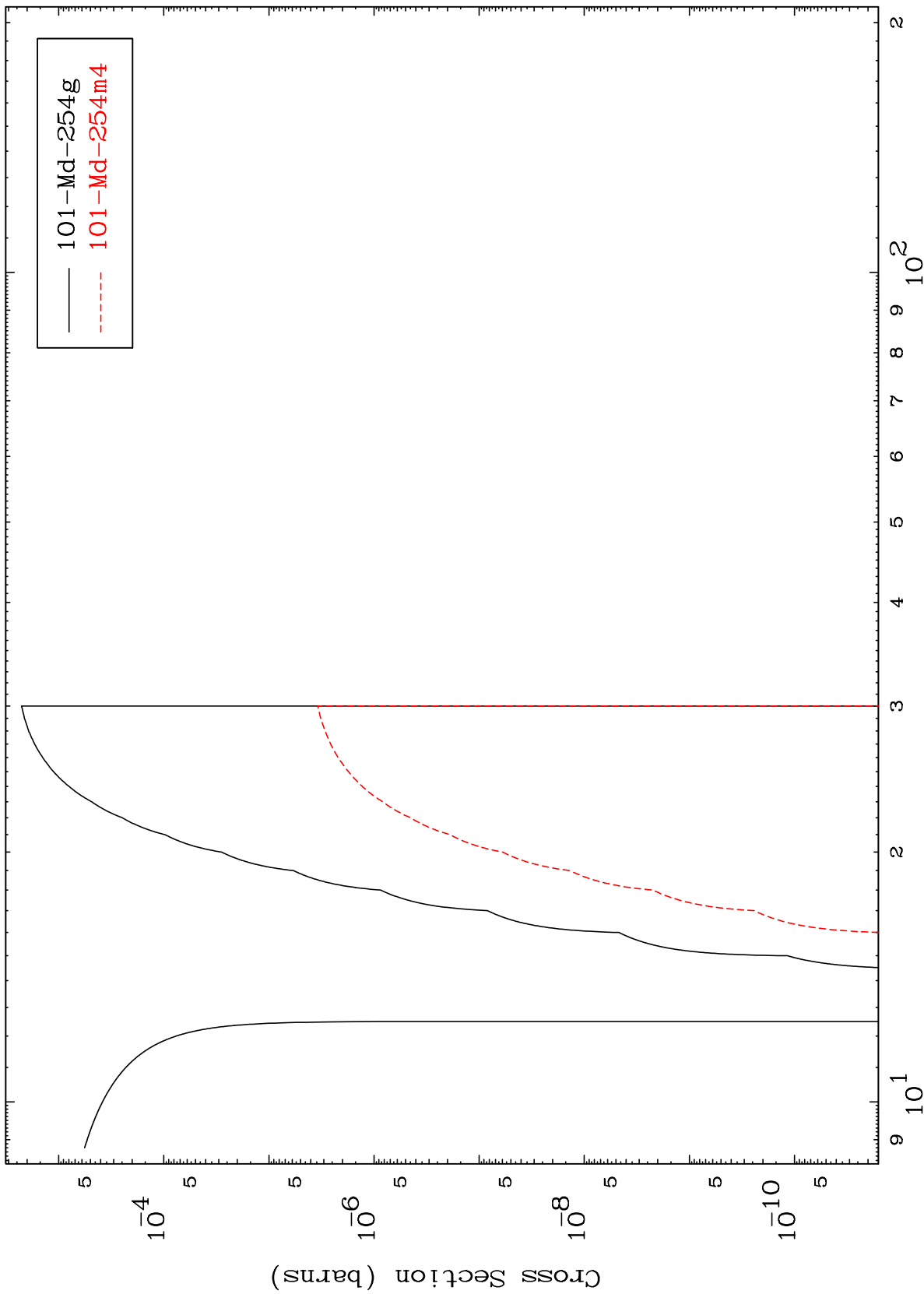


Radionuclide Production Cross Section

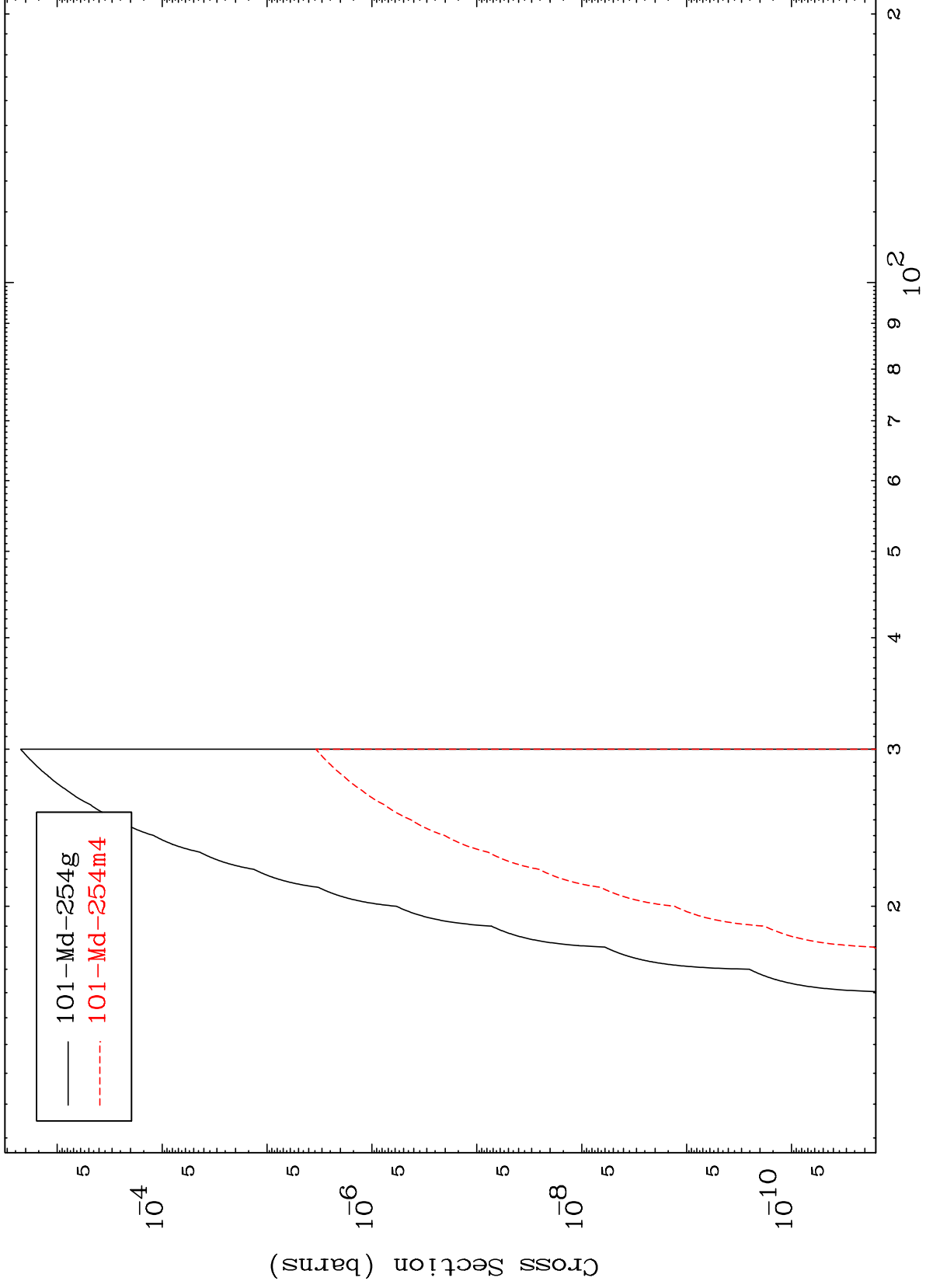


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- - - 102-No-254m12

Radionuclide Production Cross Section



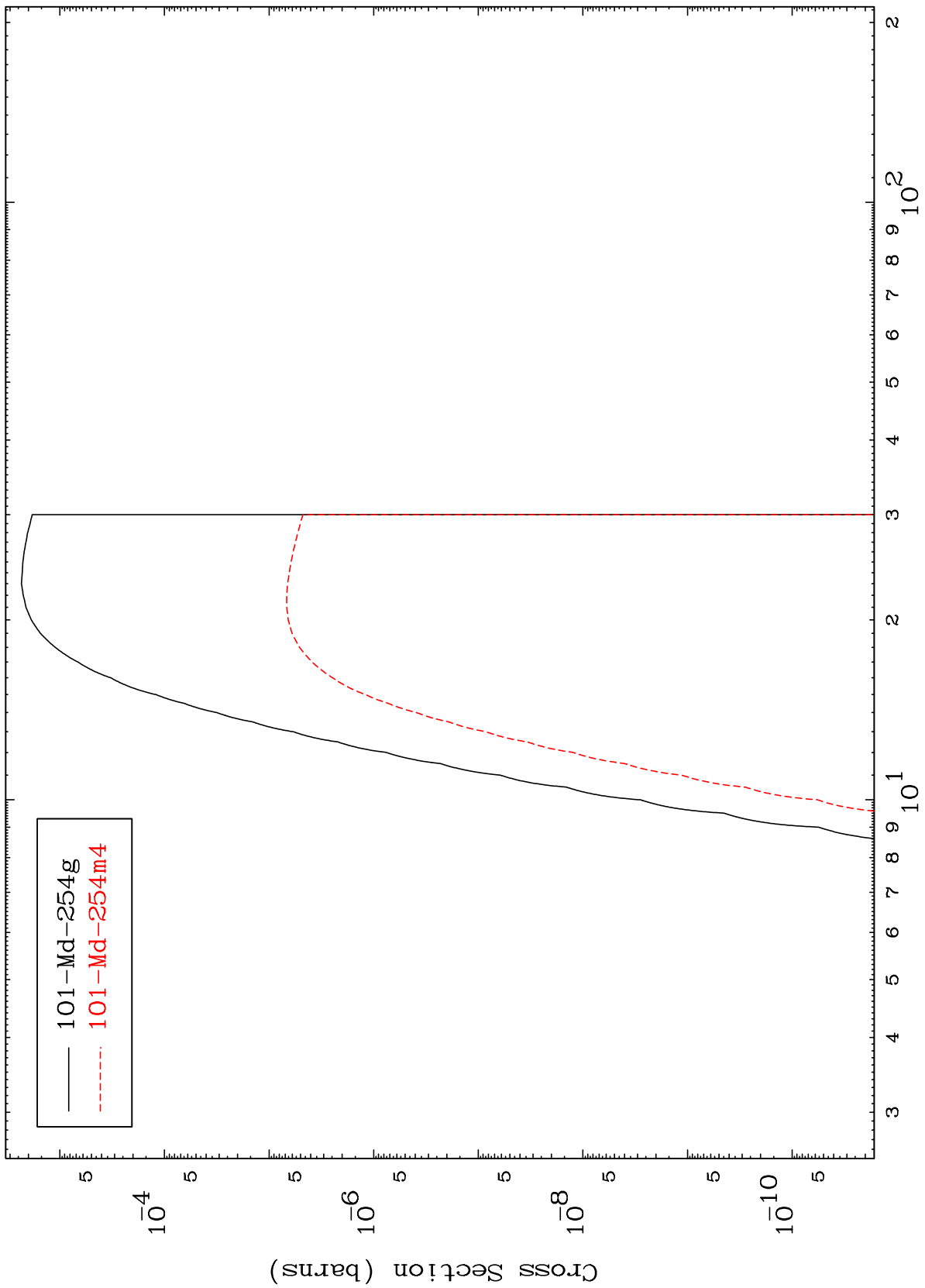
Radionuclide Production Cross Section



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102-No-256

(n, t)  
Radionuclide Production Cross Section



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Incident Energy (MeV)

102-No-256