

Program EVALPLOT
(Version 2021-1)

by

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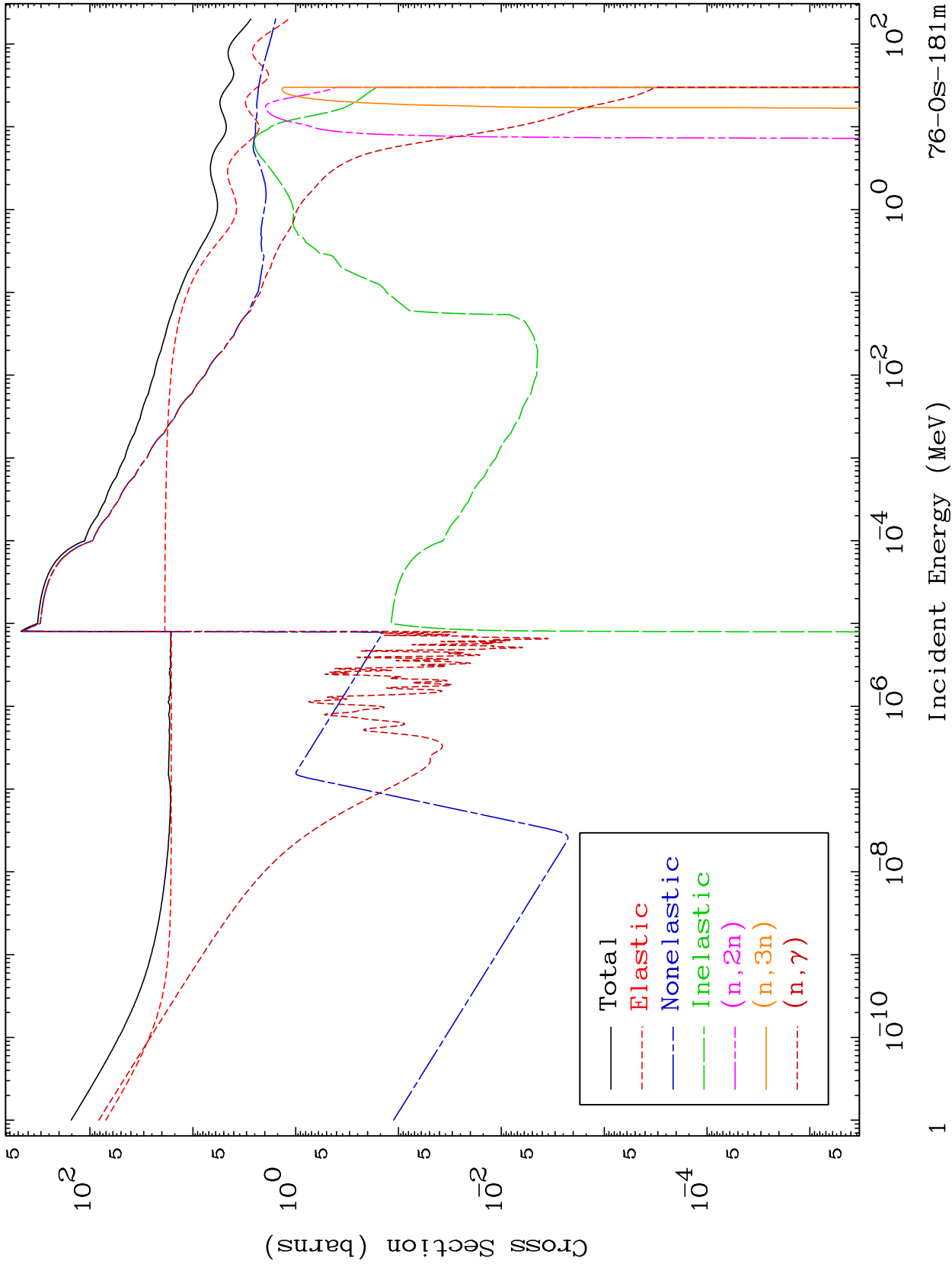
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7617

Neutron Major
293 Kelvin Cross Sections

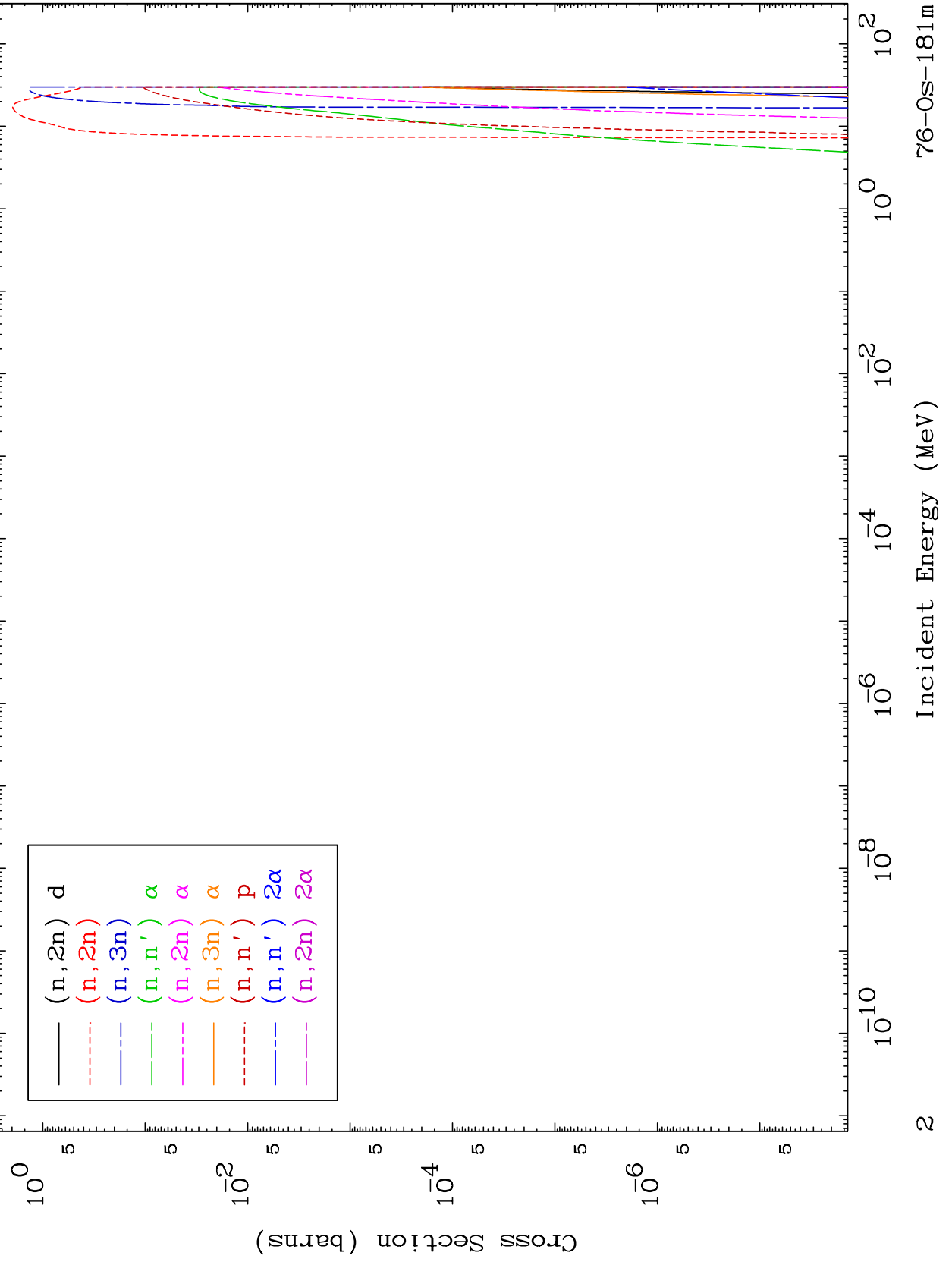
76-Os-181m



MAT 7617

Neutron Absorption
293 Kelvin Cross Sections

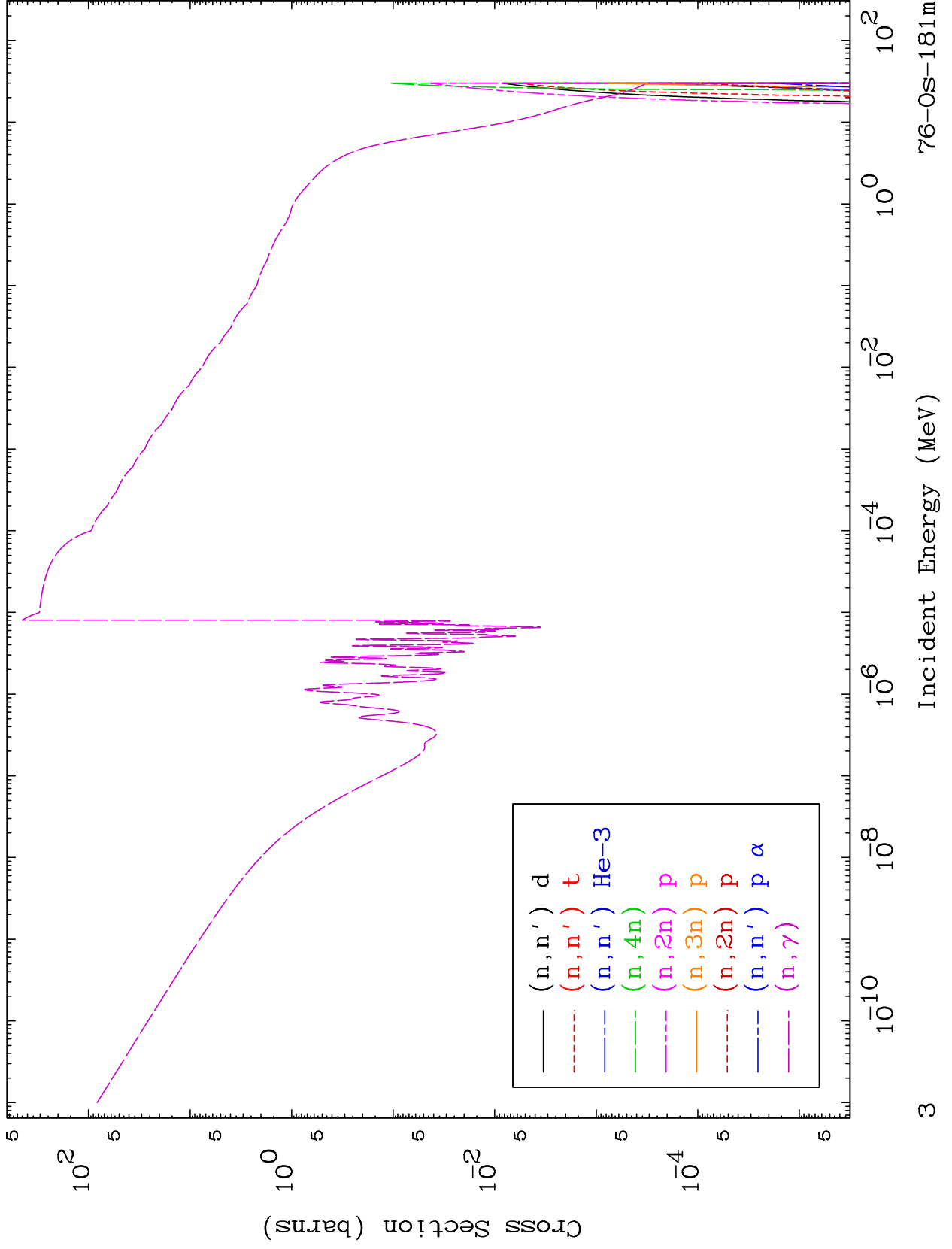
76-Os-181m



MAT 7617

Neutron Absorption
293 Kelvin Cross Sections

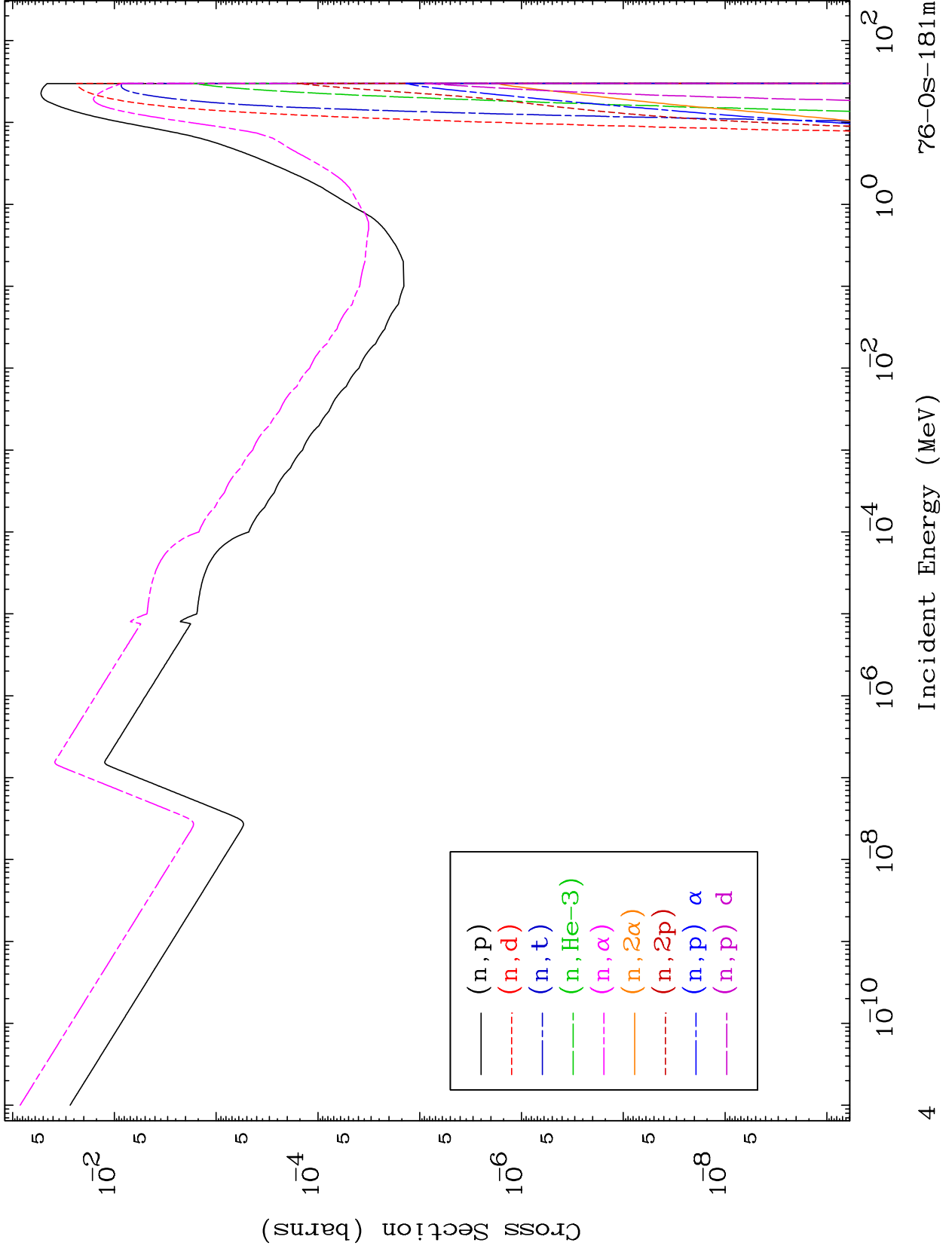
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MAT 7617

Neutron Absorption
293 Kelvin Cross Sections

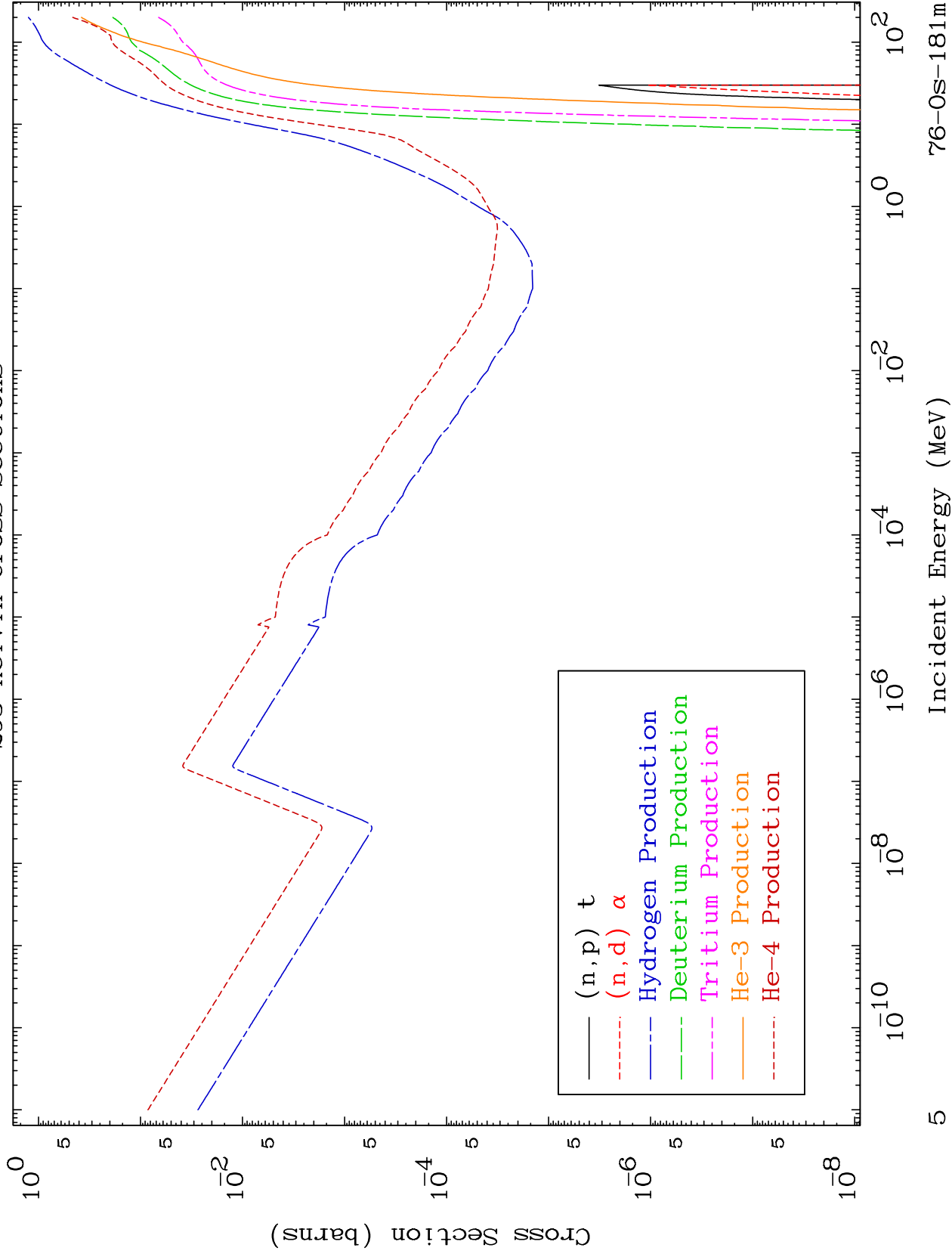
76-Os-181m



MAT 7617

Neutron Absorption
293 Kelvin Cross Sections

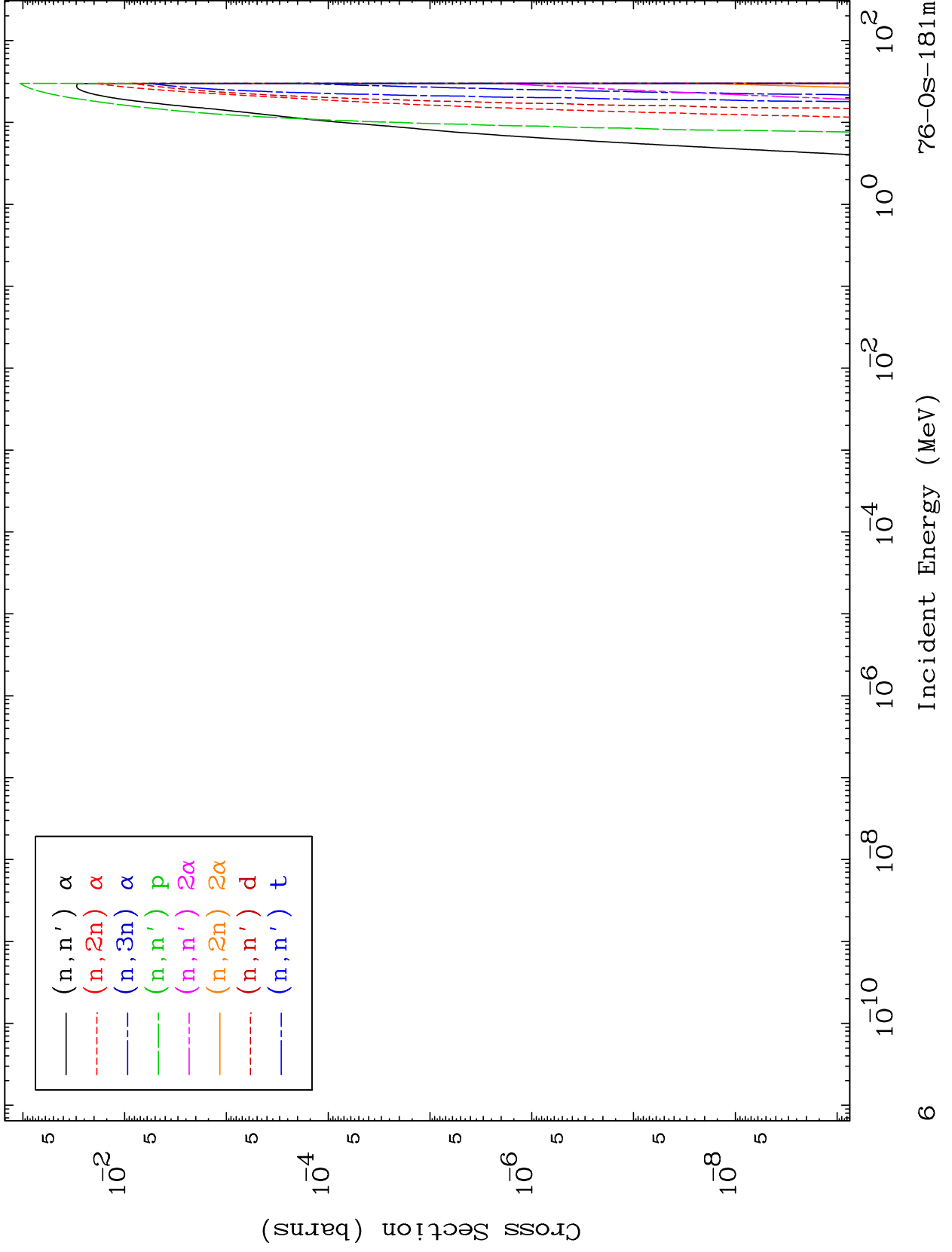
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MAT 7617

Charged Particle
293 Kelvin Cross Sections

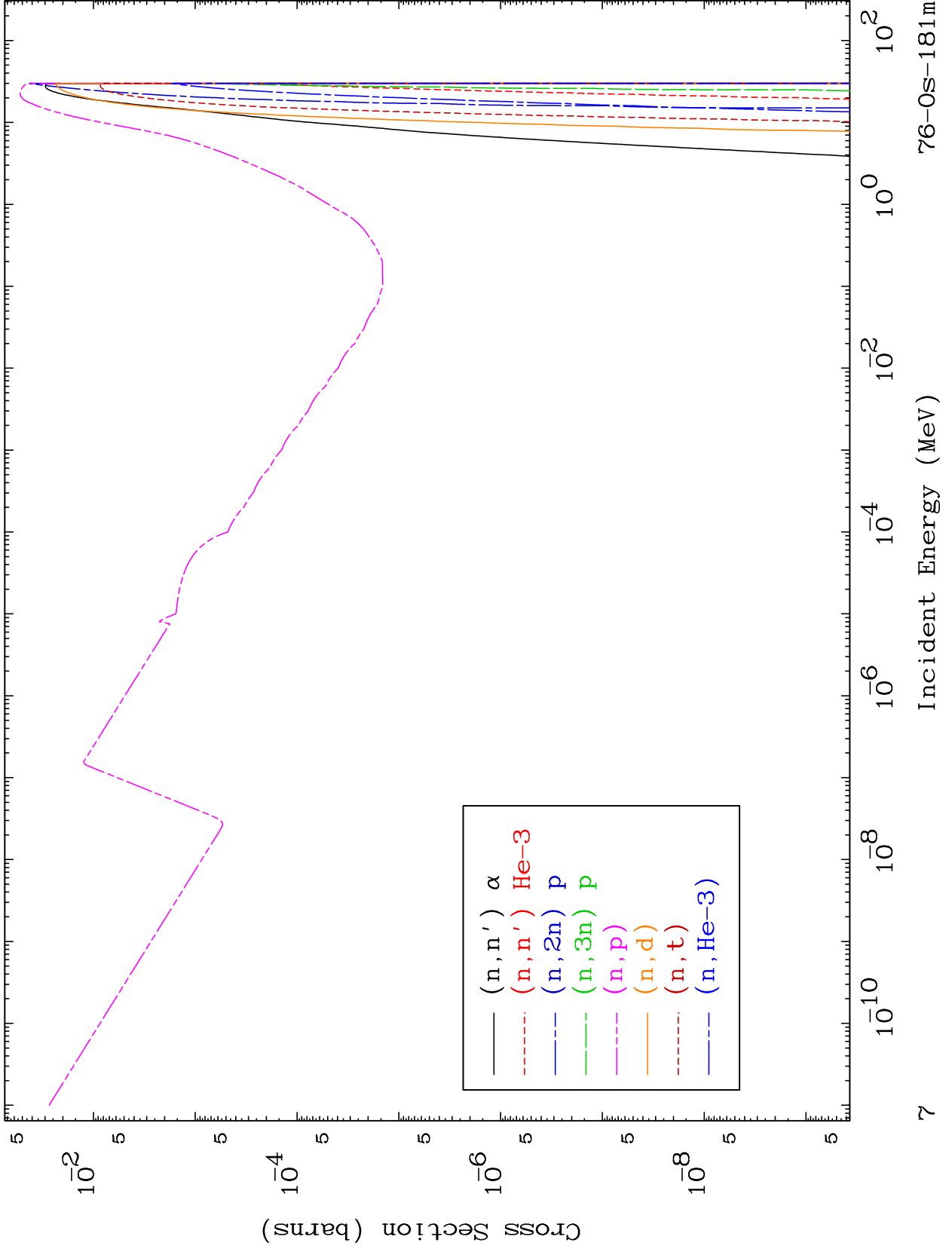
76-Os-181m



MAT 7617

Charged Particle
293 Kelvin Cross Sections

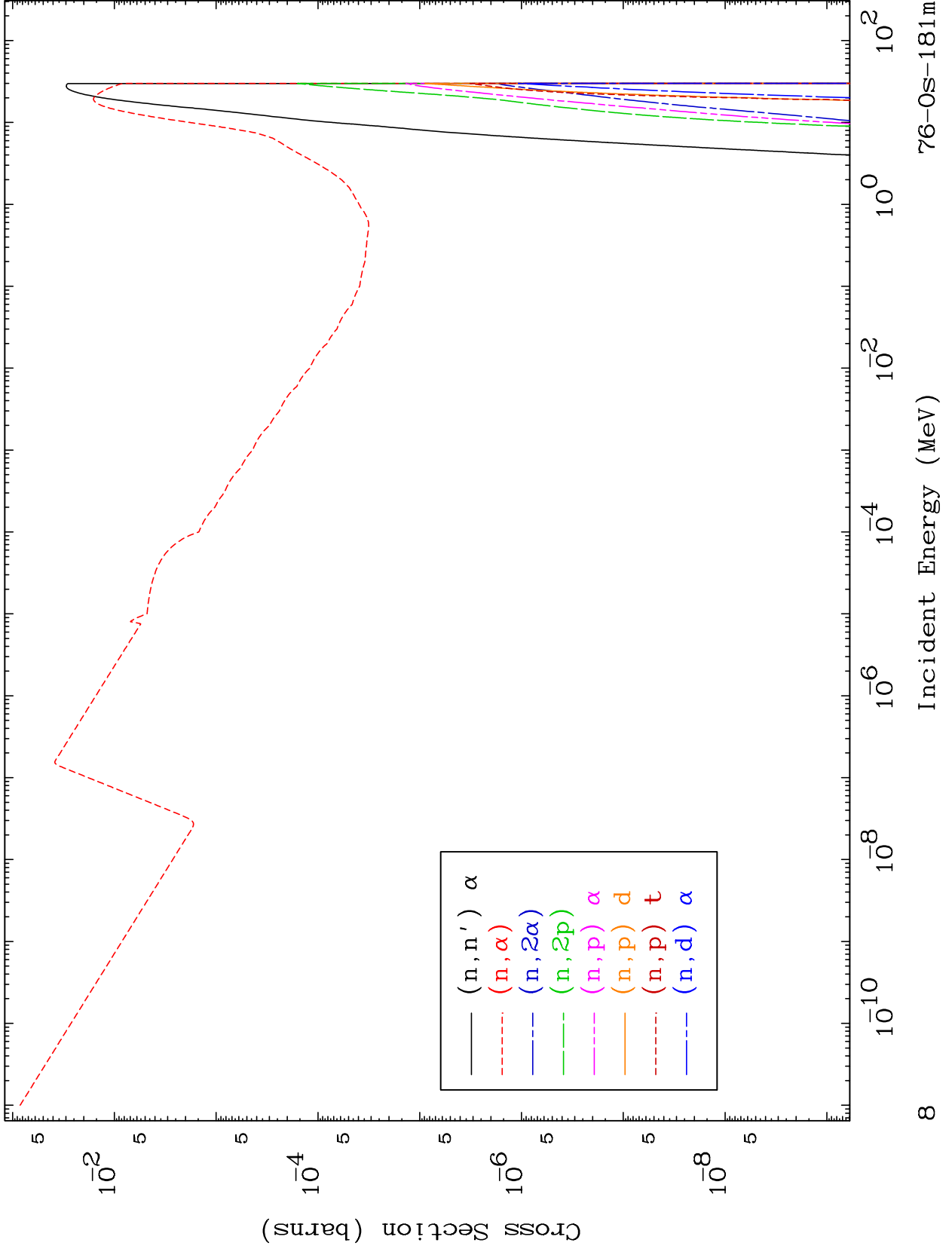
76-Os-181m



MAT 7617

Charged Particle
293 Kelvin Cross Sections

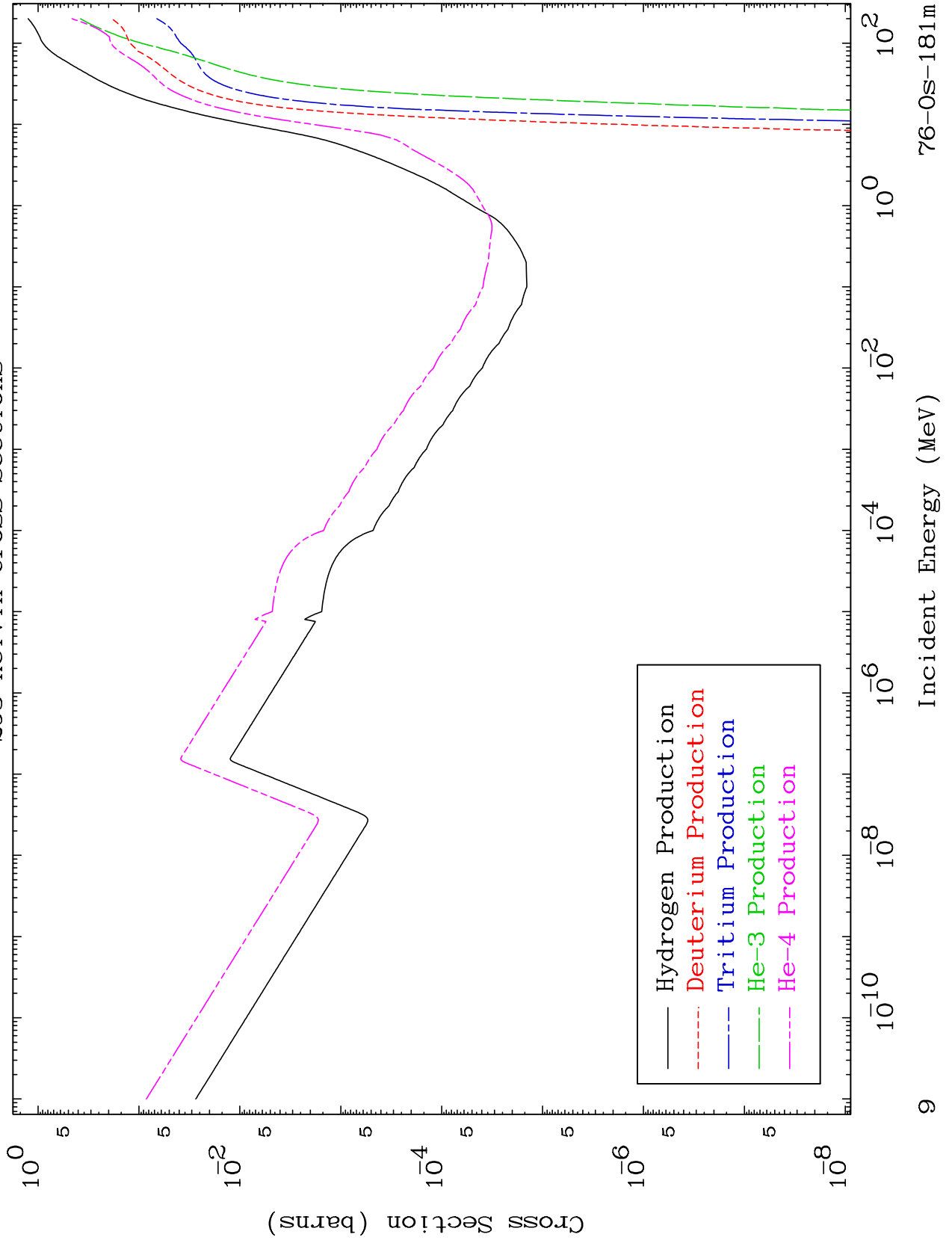
76-Os-181m



MAT 7617

Particle Production
293 Kelvin Cross Sections

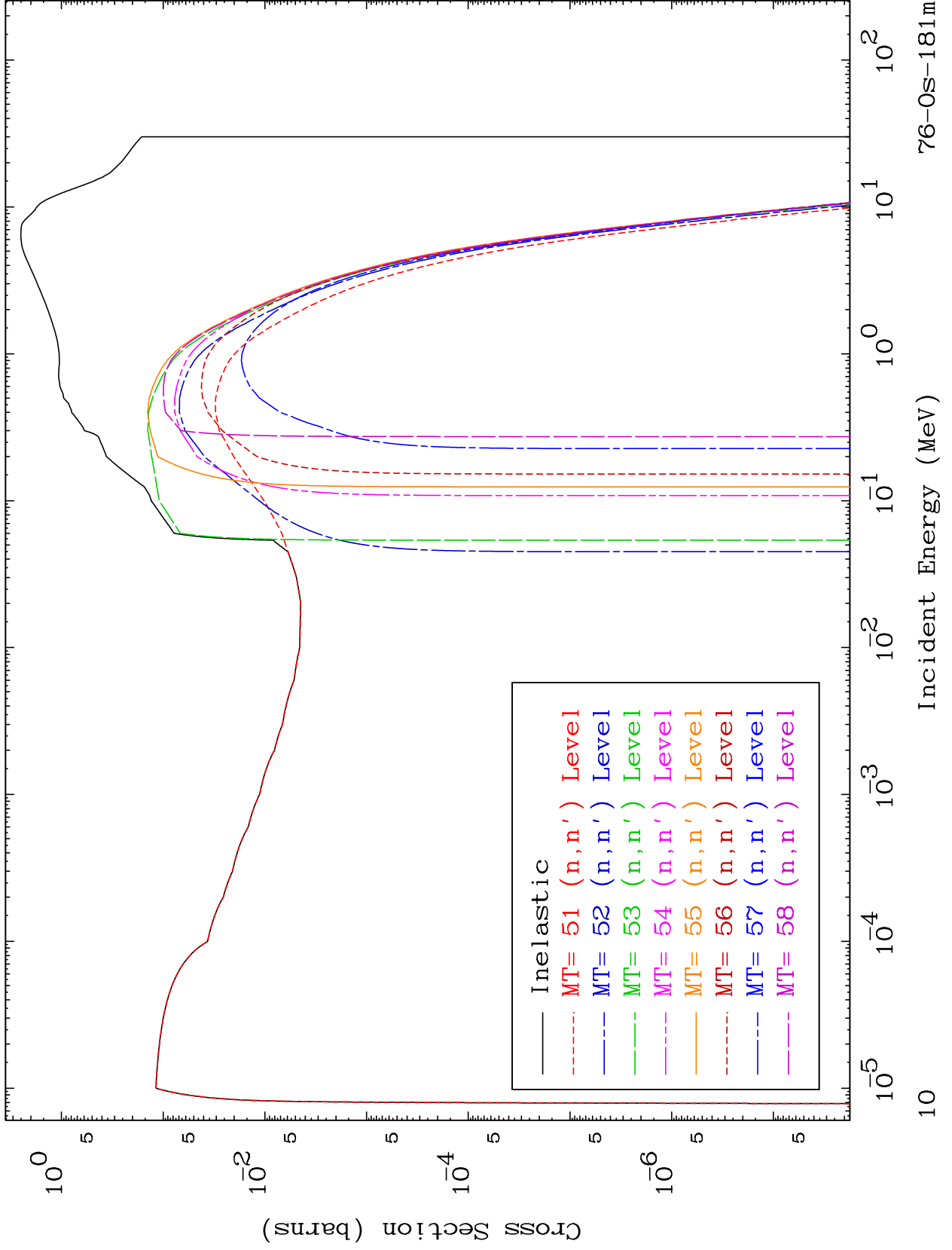
76-Os-181m



MAT 7617

(n,n') Levels
293 Kelvin Cross Sections

76-0s-181m



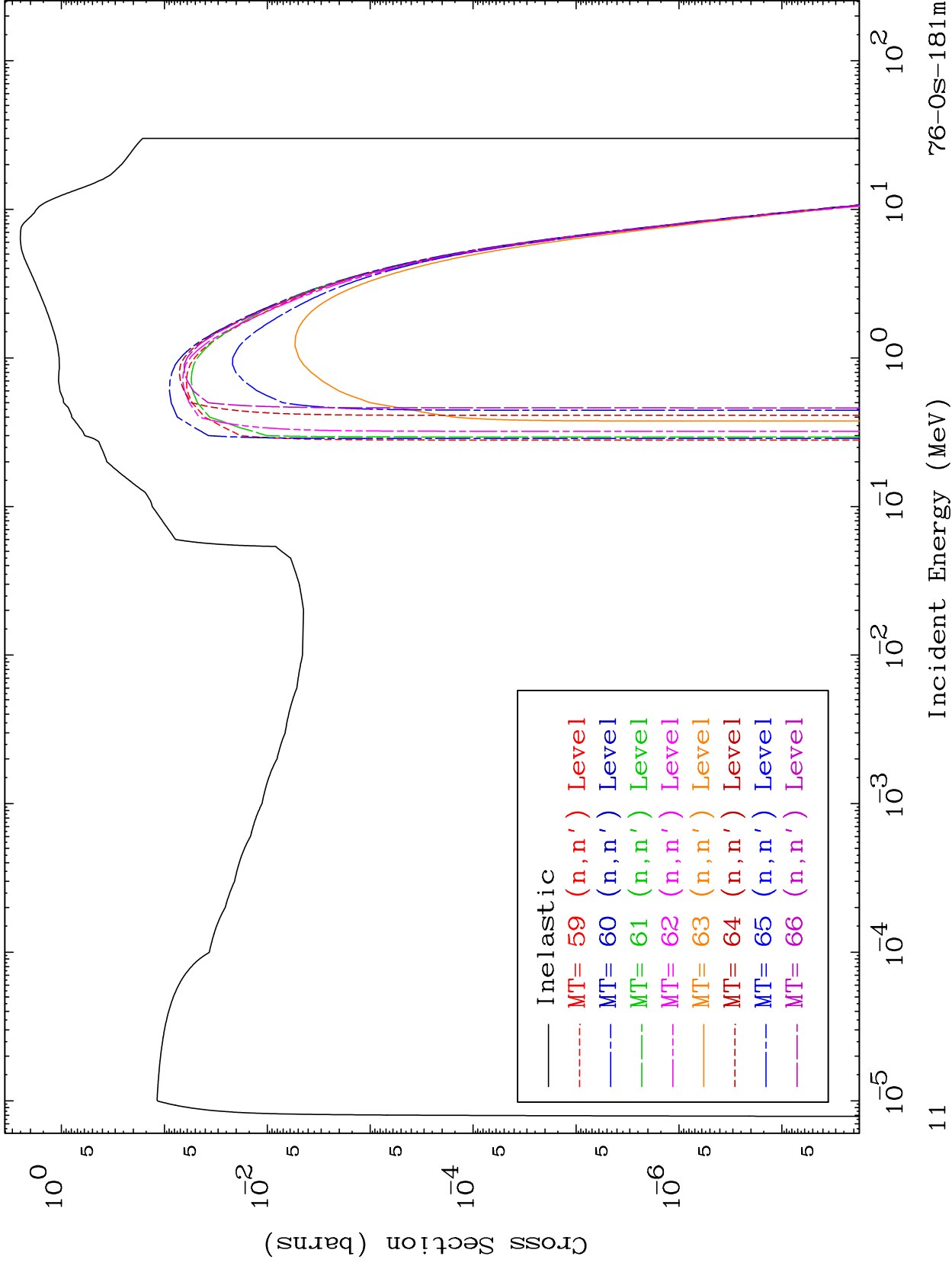
76-0s-181m

Incident Energy (MeV)

MAT 7617

(n,n') Levels
293 Kelvin Cross Sections

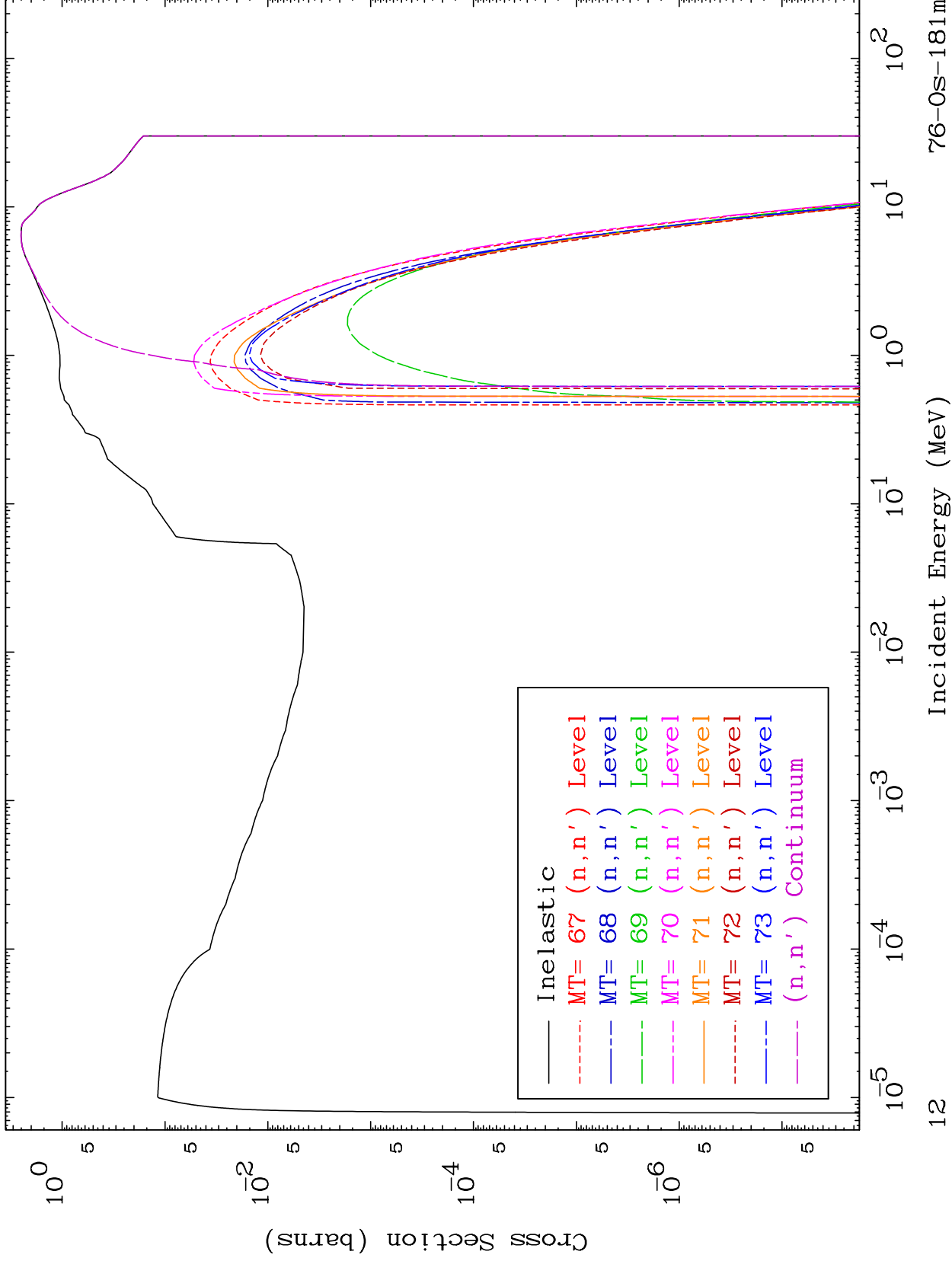
76-0s-181m



MAT 7617

(n,n') Levels
293 Kelvin Cross Sections

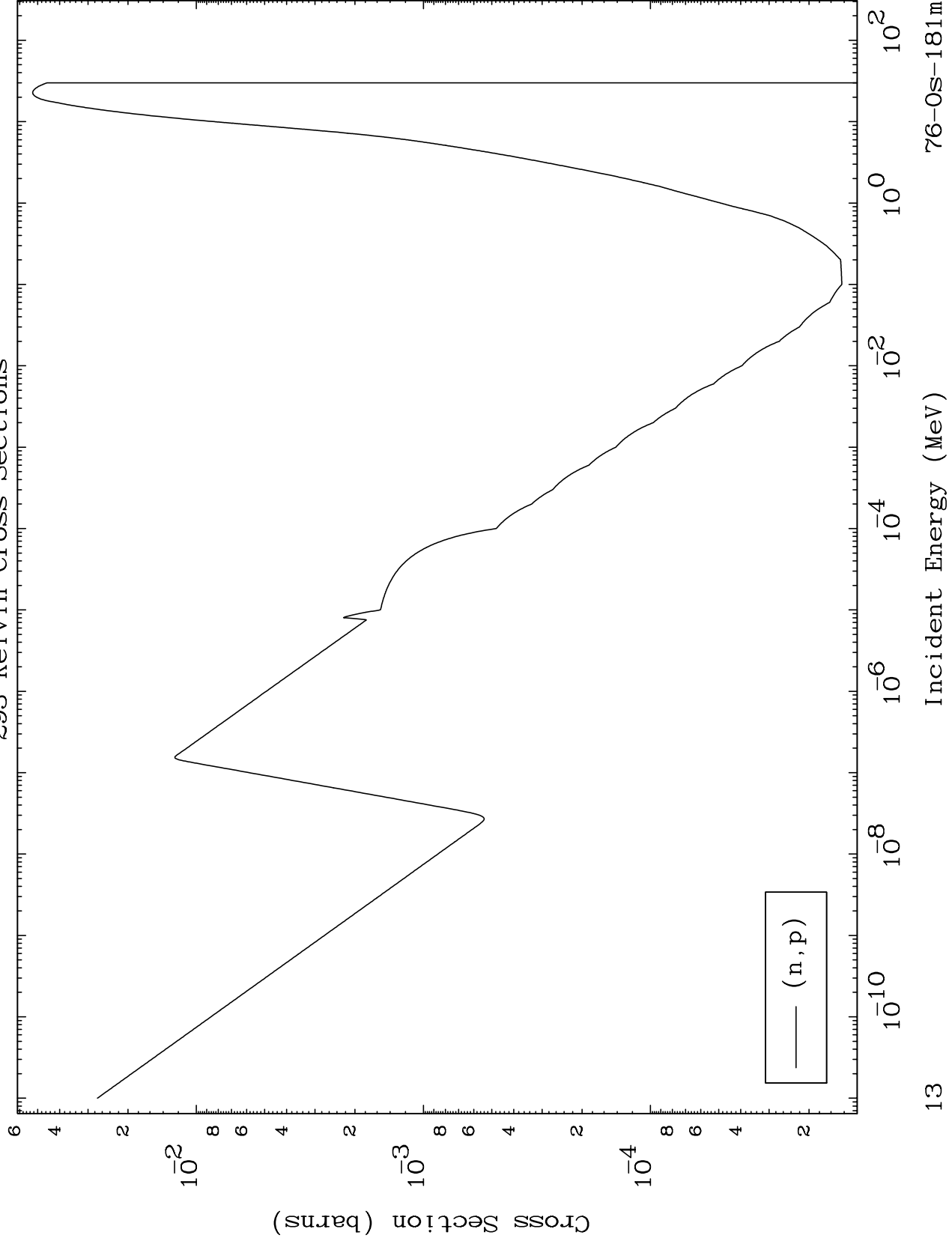
76-0s-181m



MAT 7617

(n,p) Levels
293 Kelvin Cross Sections

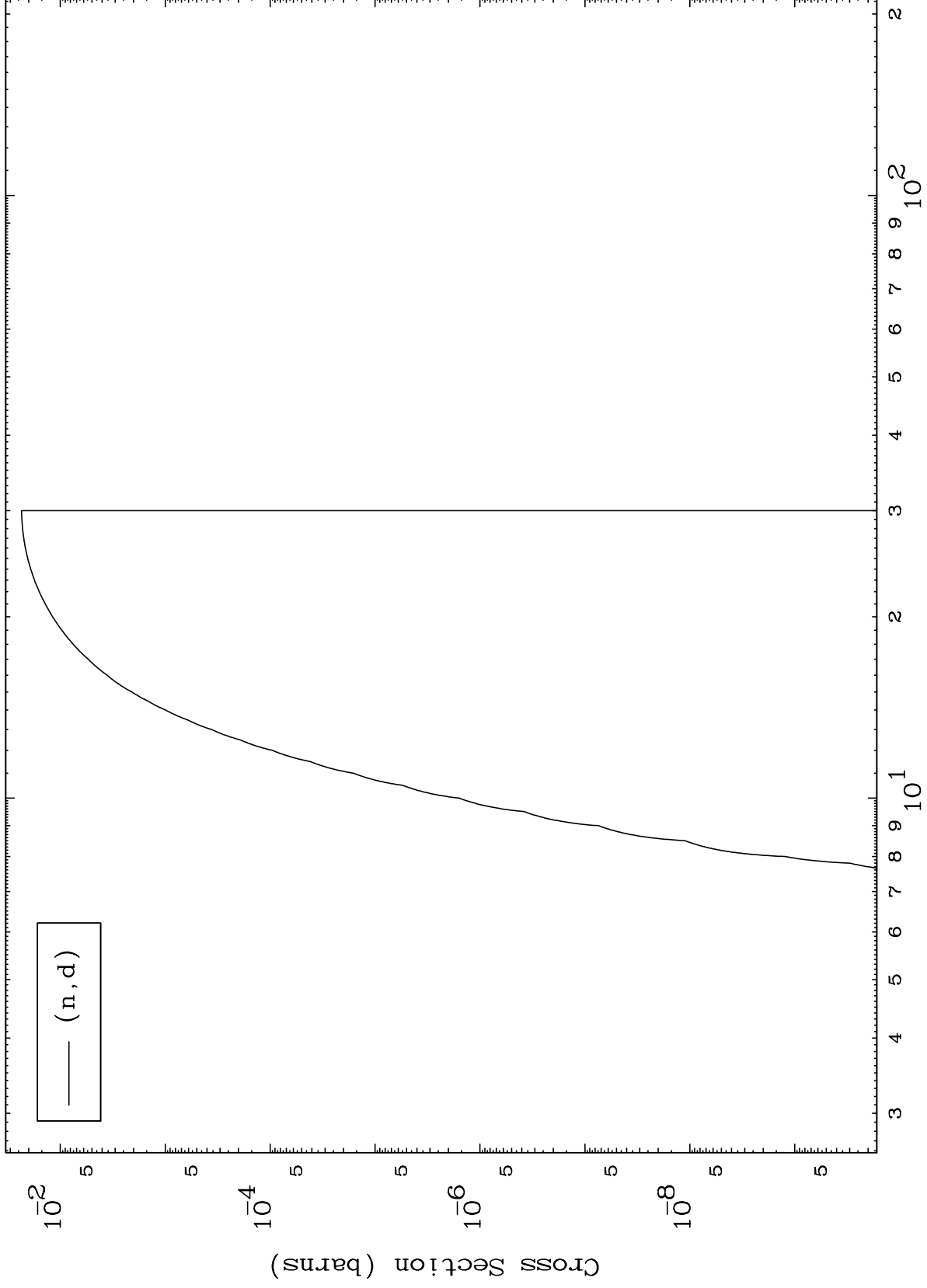
76-Os-181m



MAT 7617

(n,d) Levels
293 Kelvin Cross Sections

76-0s-181m



14

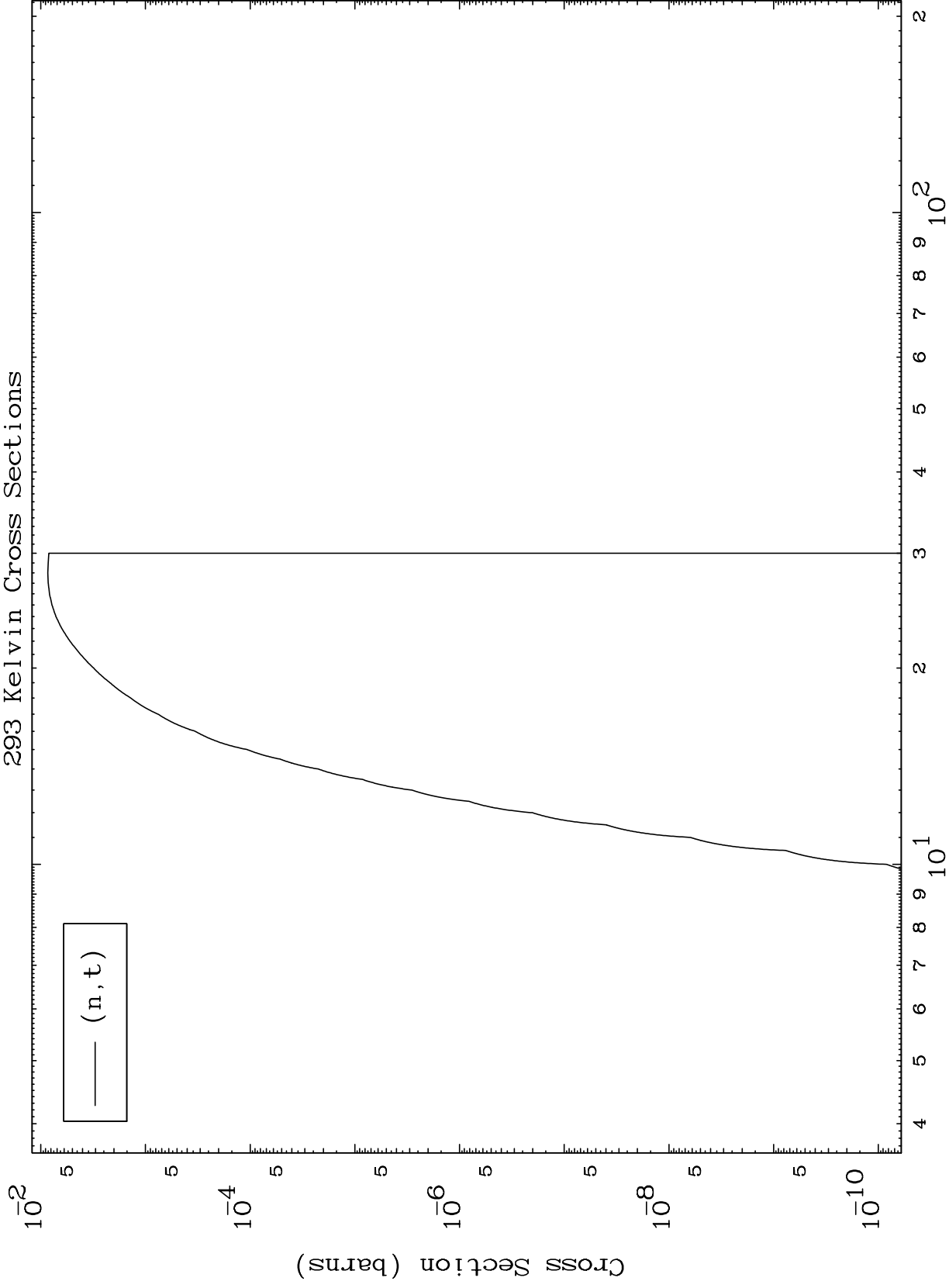
Incident Energy (MeV)

76-0s-181m

MAT 7617

(n,t) Levels
293 Kelvin Cross Sections

76-0s-181m



15

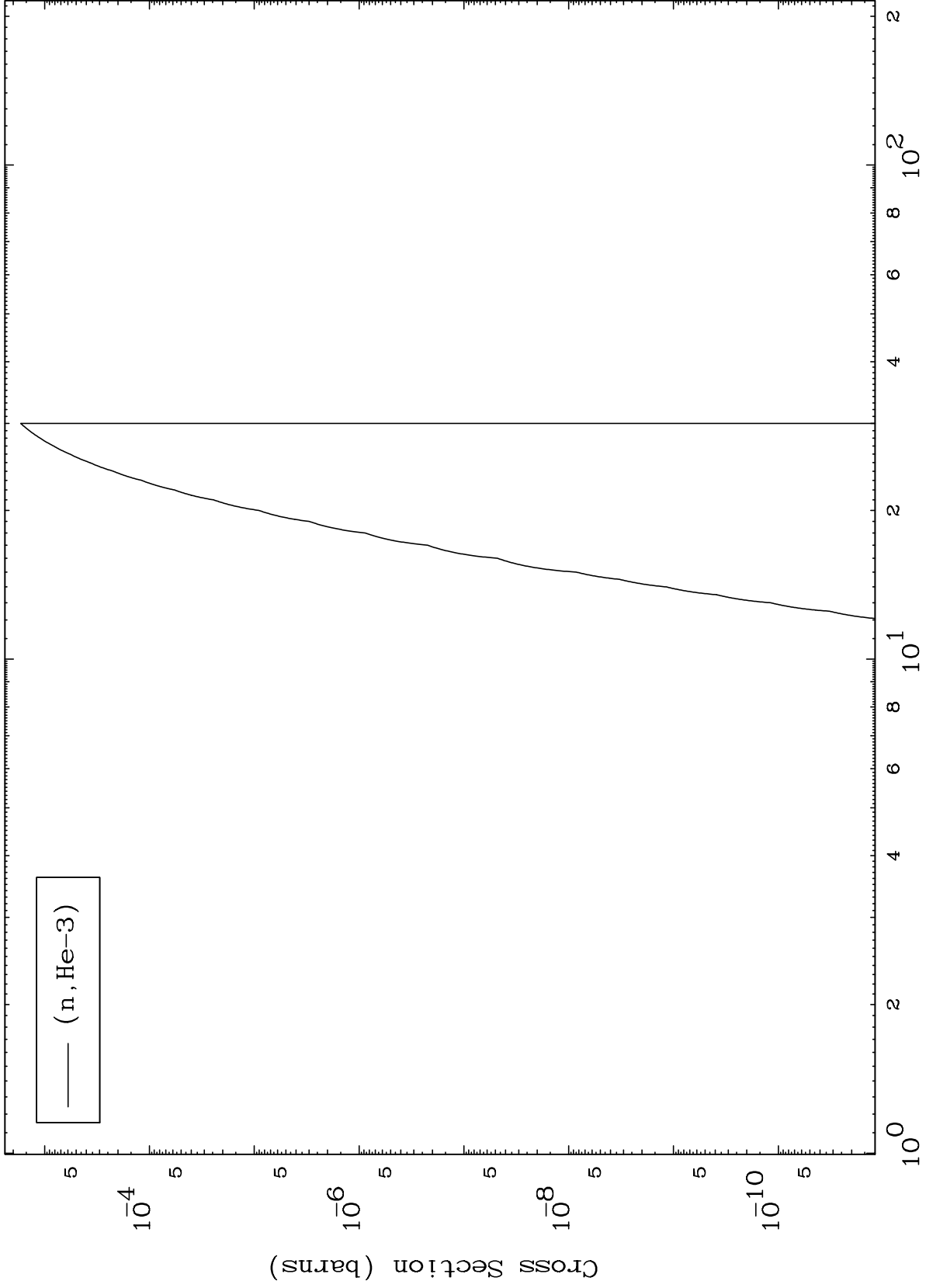
Incident Energy (MeV)

76-0s-181m

MAT 7617

(n,He3) Levels
293 Kelvin Cross Sections

76-0s-181m



16

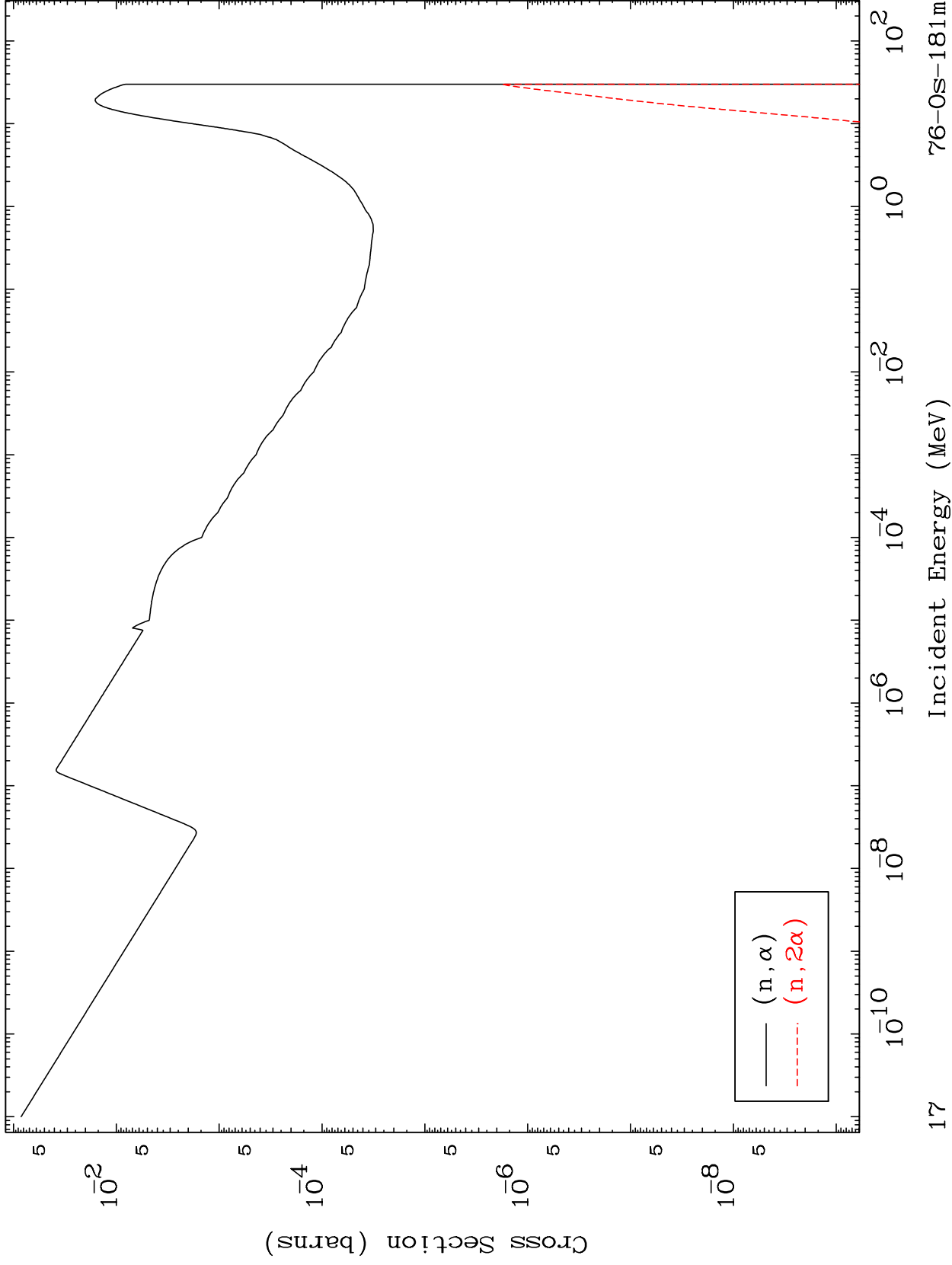
Incident Energy (MeV)

76-0s-181m

MAT 7617

(n,α) Levels
293 Kelvin Cross Sections

76-0s-181m



MAT 7617

Elastic Legendre Coefficients

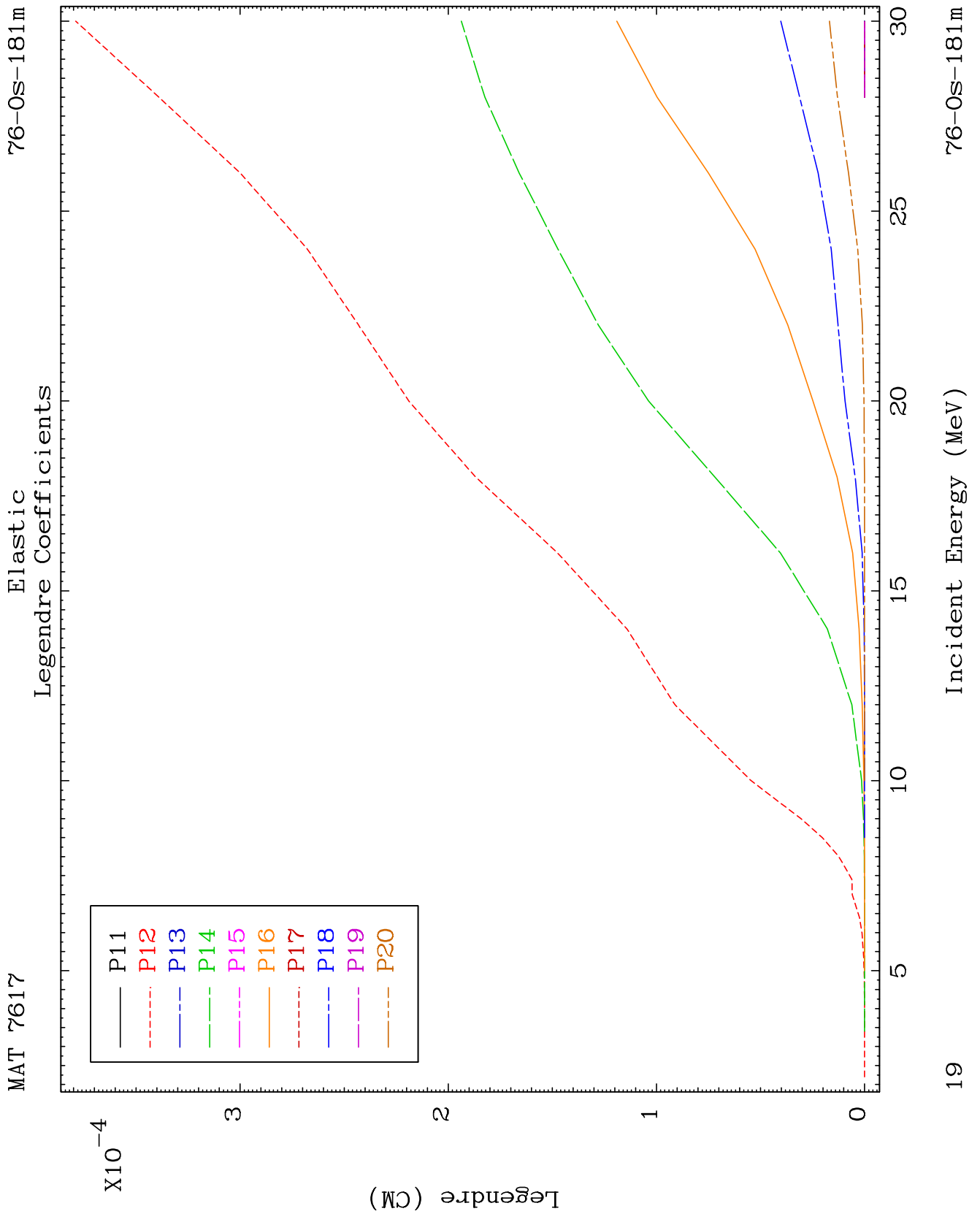
76-0s-181m



18

Incident Energy (MeV)

76-0s-181m



MAT 7617

Elastic Legendre Coefficients

76-0s-181m

- P11
- P12
- P13
- P14
- P15
- P16
- P17
- P18
- P19
- P20

$\times 10^{-4}$

Legendre (CM)

19

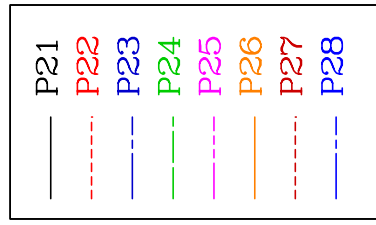
Incident Energy (MeV)

76-0s-181m

MAT 7617

Elastic
Legendre Coefficients

76-0s-181m



$\times 10^{-6}$
1.5

Legendre (CM)

1.0

0.5

0.0

15

20

25

30

20

Incident Energy (MeV)

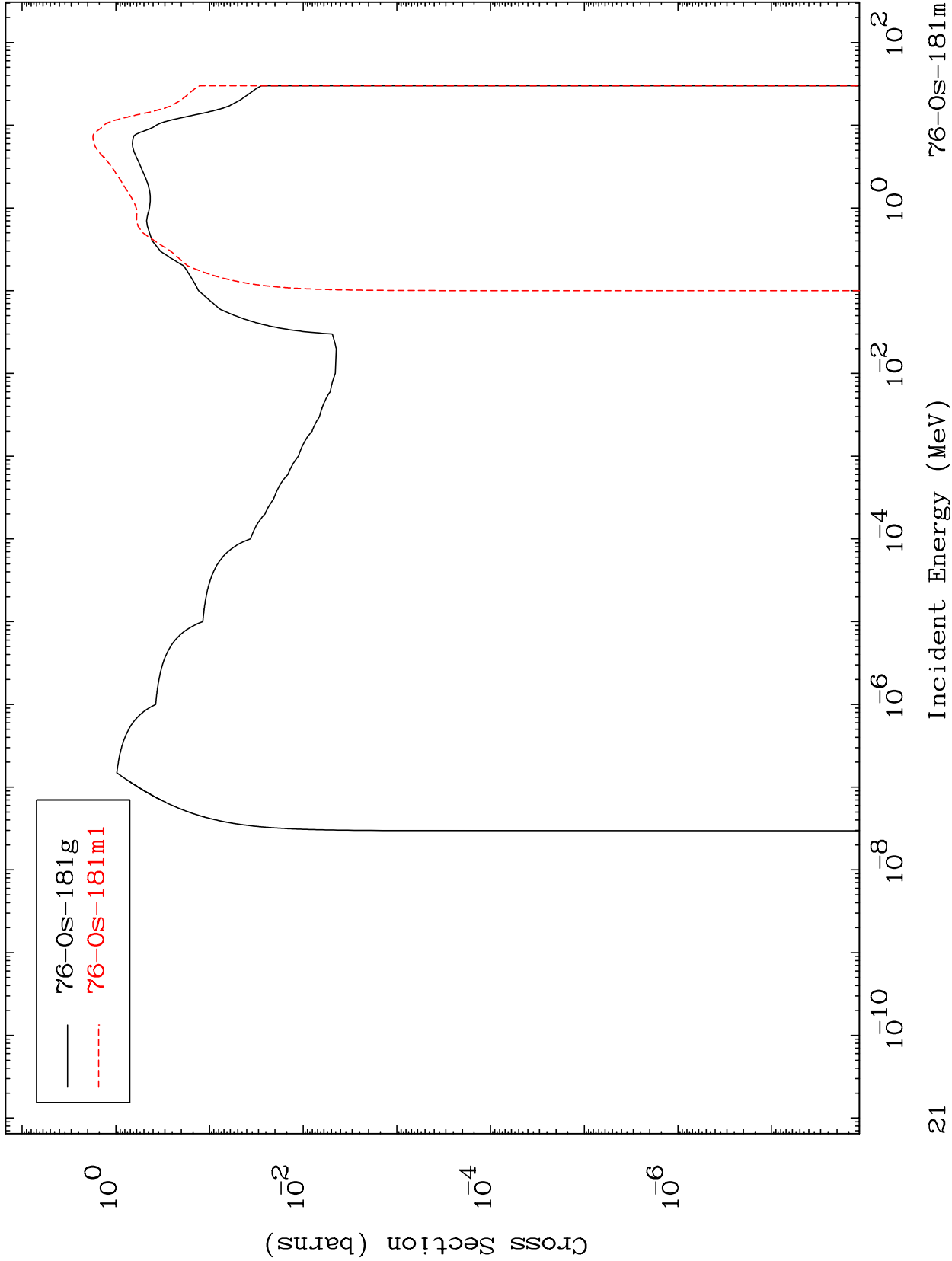
76-0s-181m



MAT 7617

Inelastic
Radionuclide Production Cross Section

76-0s-181m

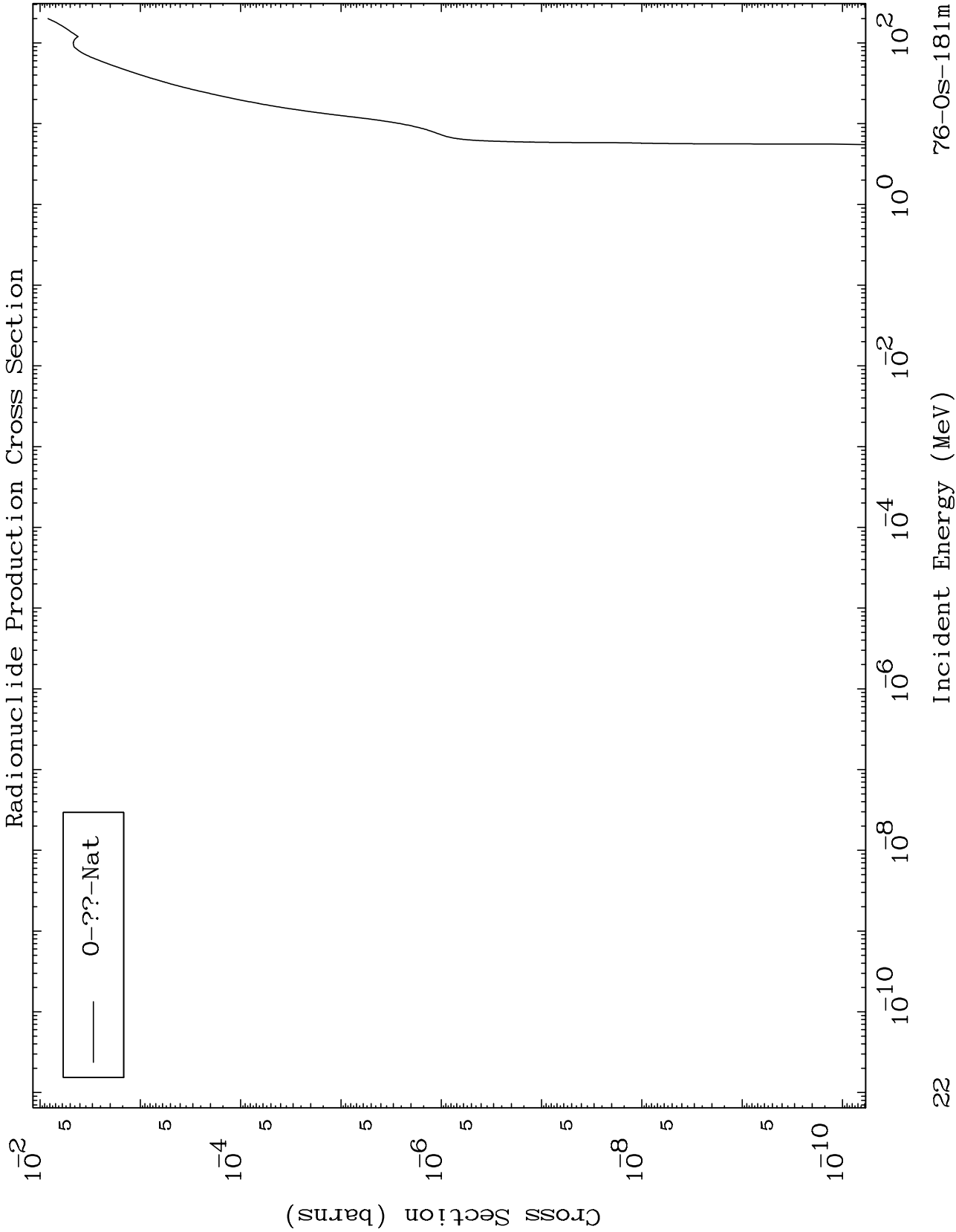


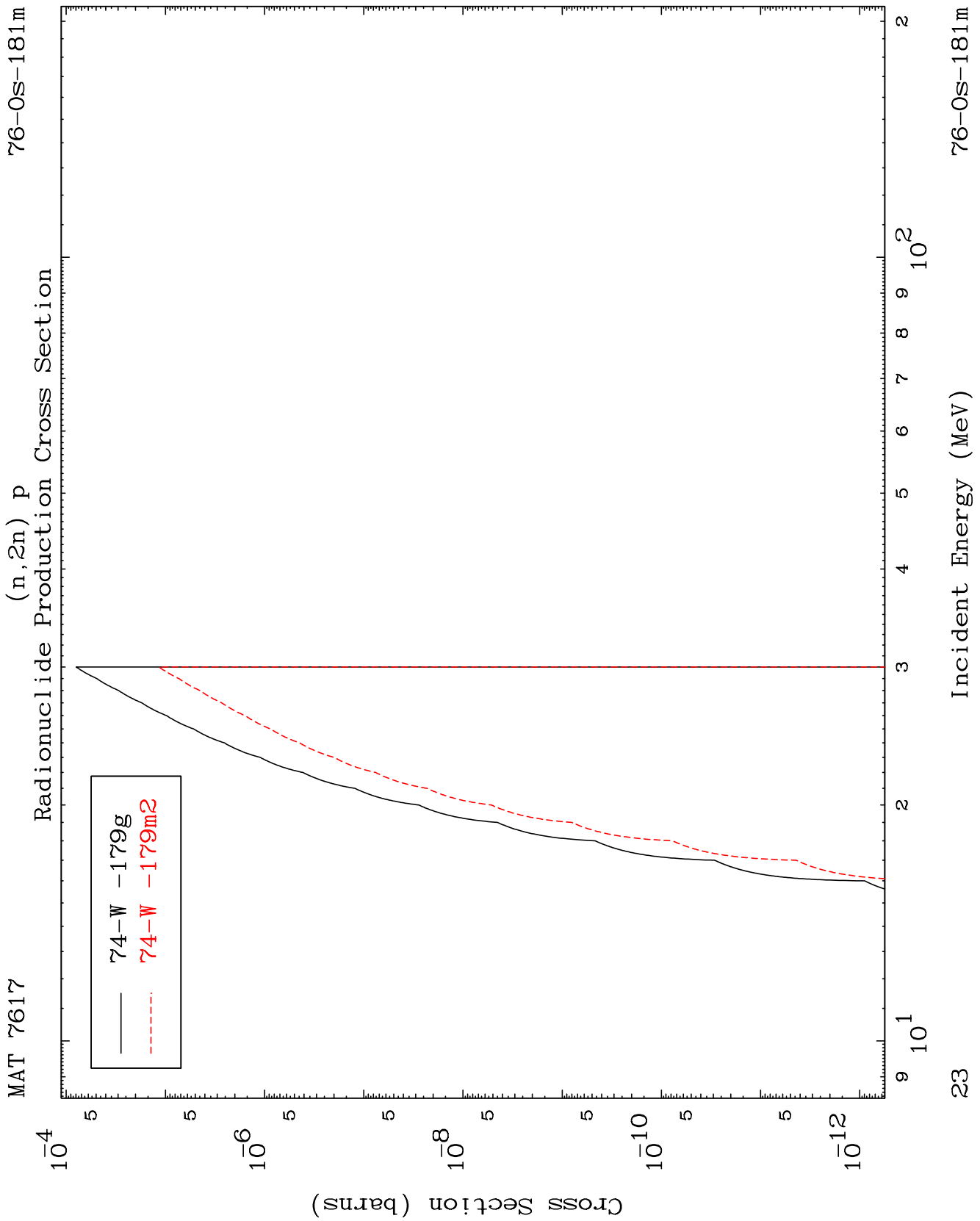
MAT 7617

Fission

76-0s-181m

Radionuclide Production Cross Section



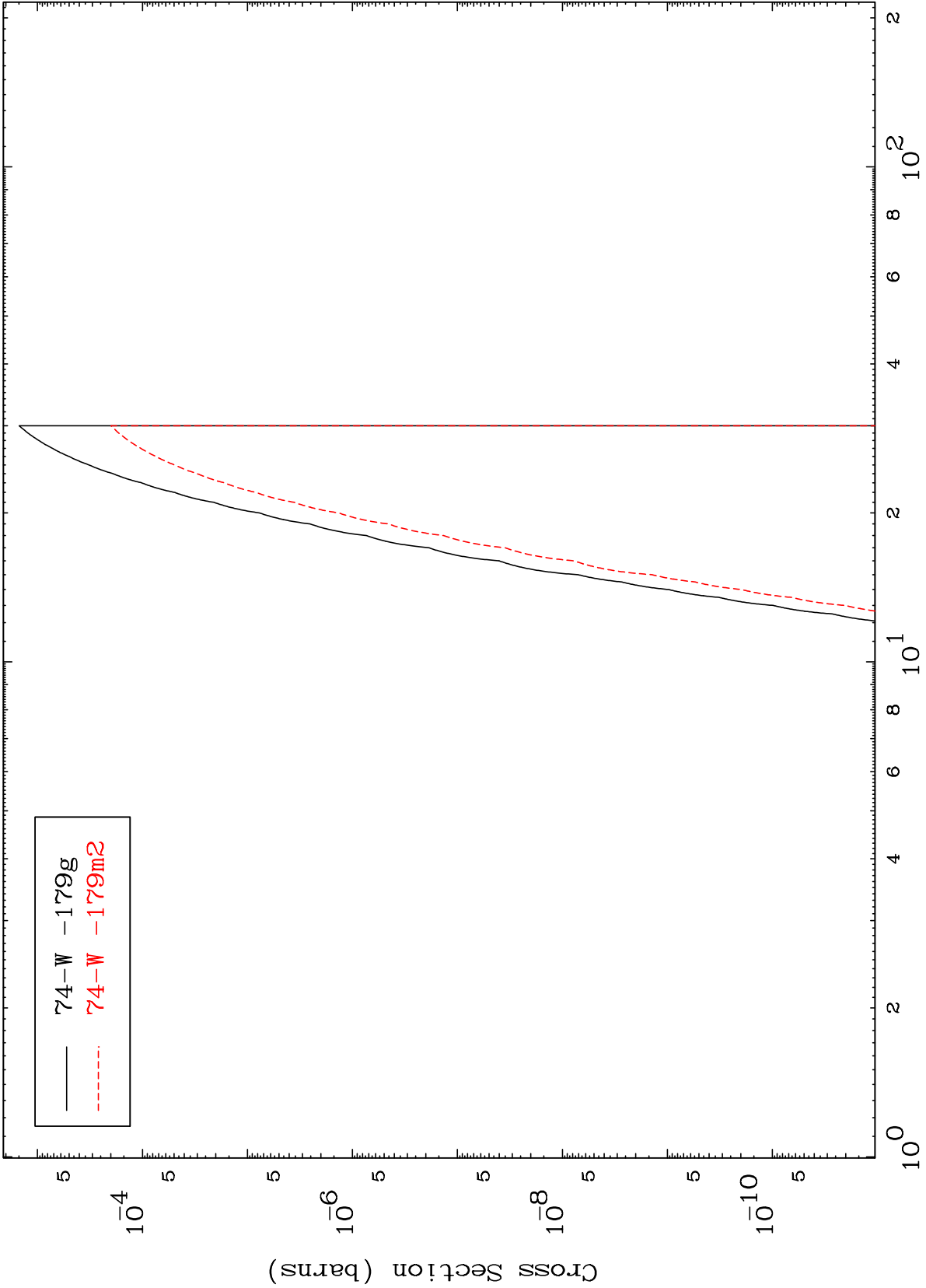


MAT 7617

(n,He-3)

76-0s-181m

Radionuclide Production Cross Section



Incident Energy (MeV)

76-0s-181m

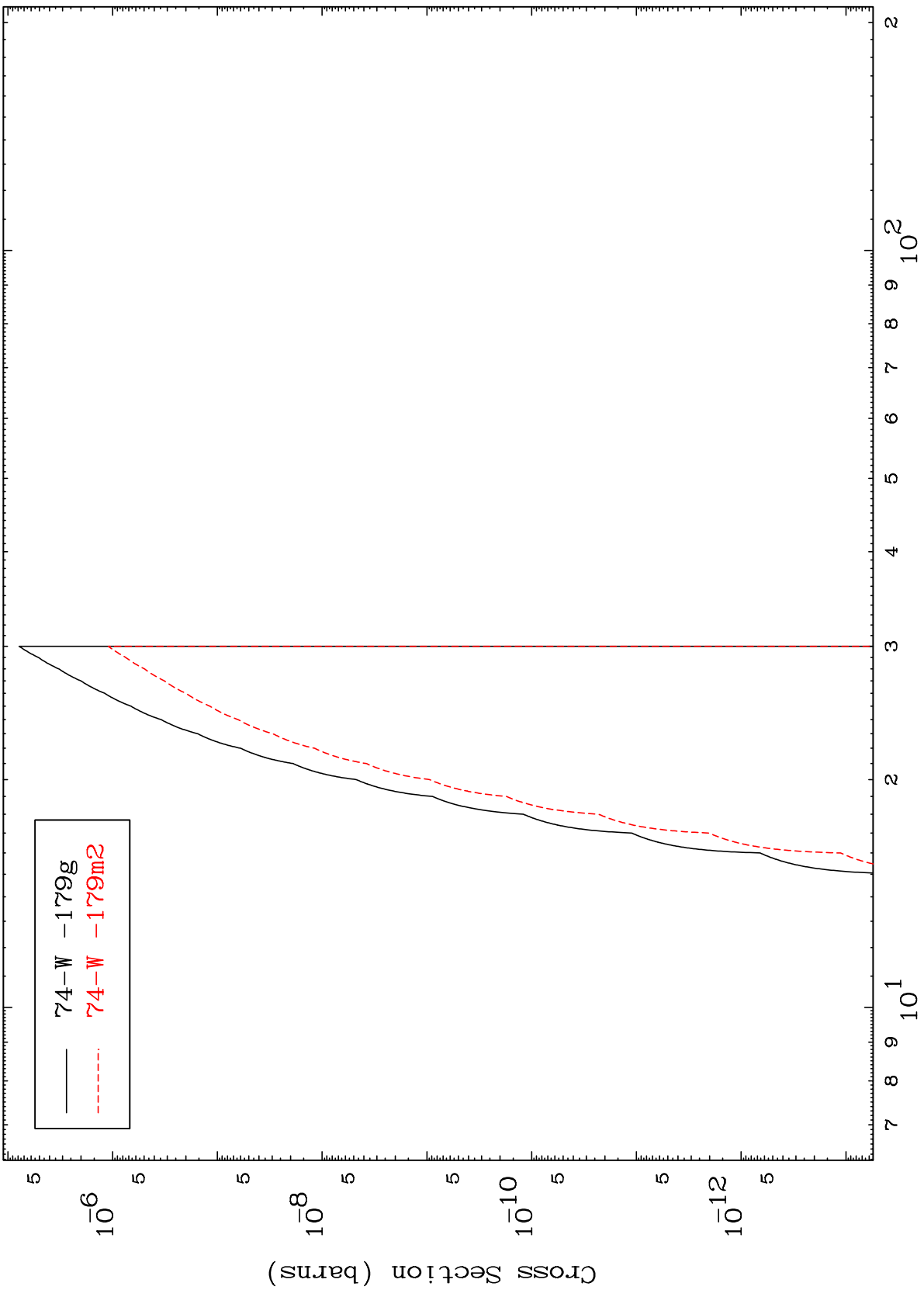
24

MAT 7617

(n,p) d

76-0s-181m

Radionuclide Production Cross Section



25

Incident Energy (MeV)

76-0s-181m