

Program EVALPLOT
(Version 2021-1)

by

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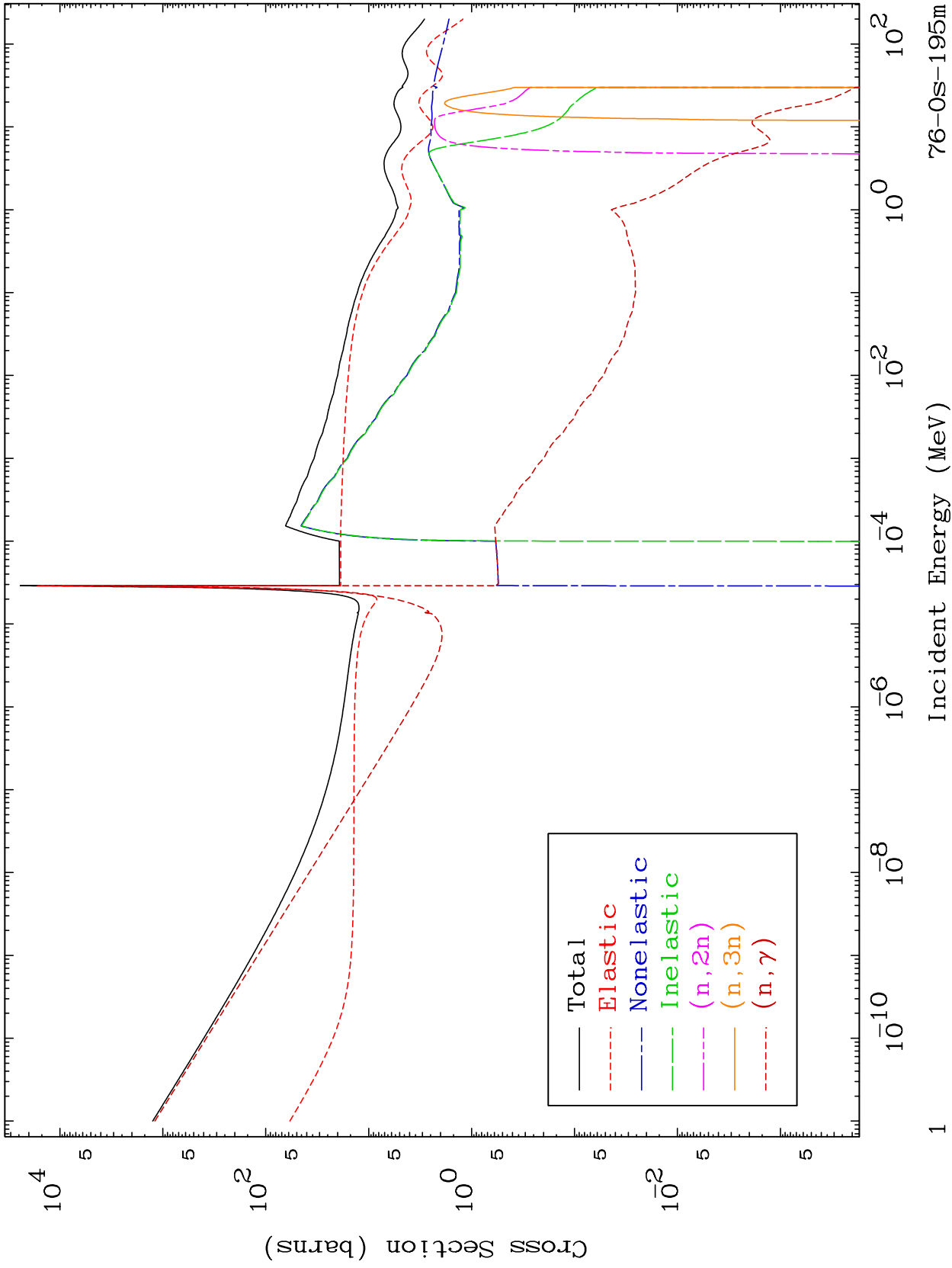
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7659

Neutron Major
293 Kelvin Cross Sections

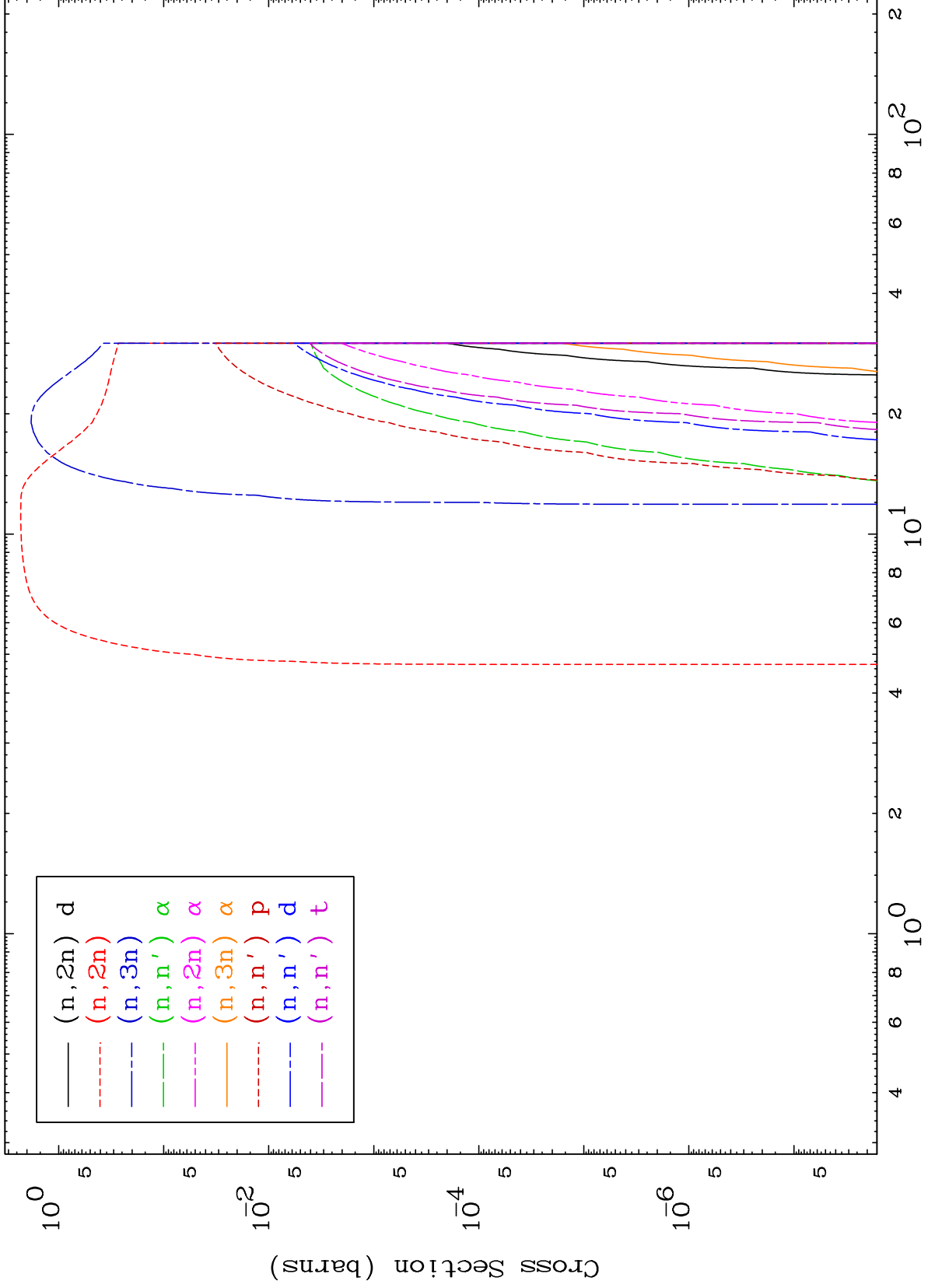
76-0s-195m



MAT 7659

Neutron Absorption
293 Kelvin Cross Sections

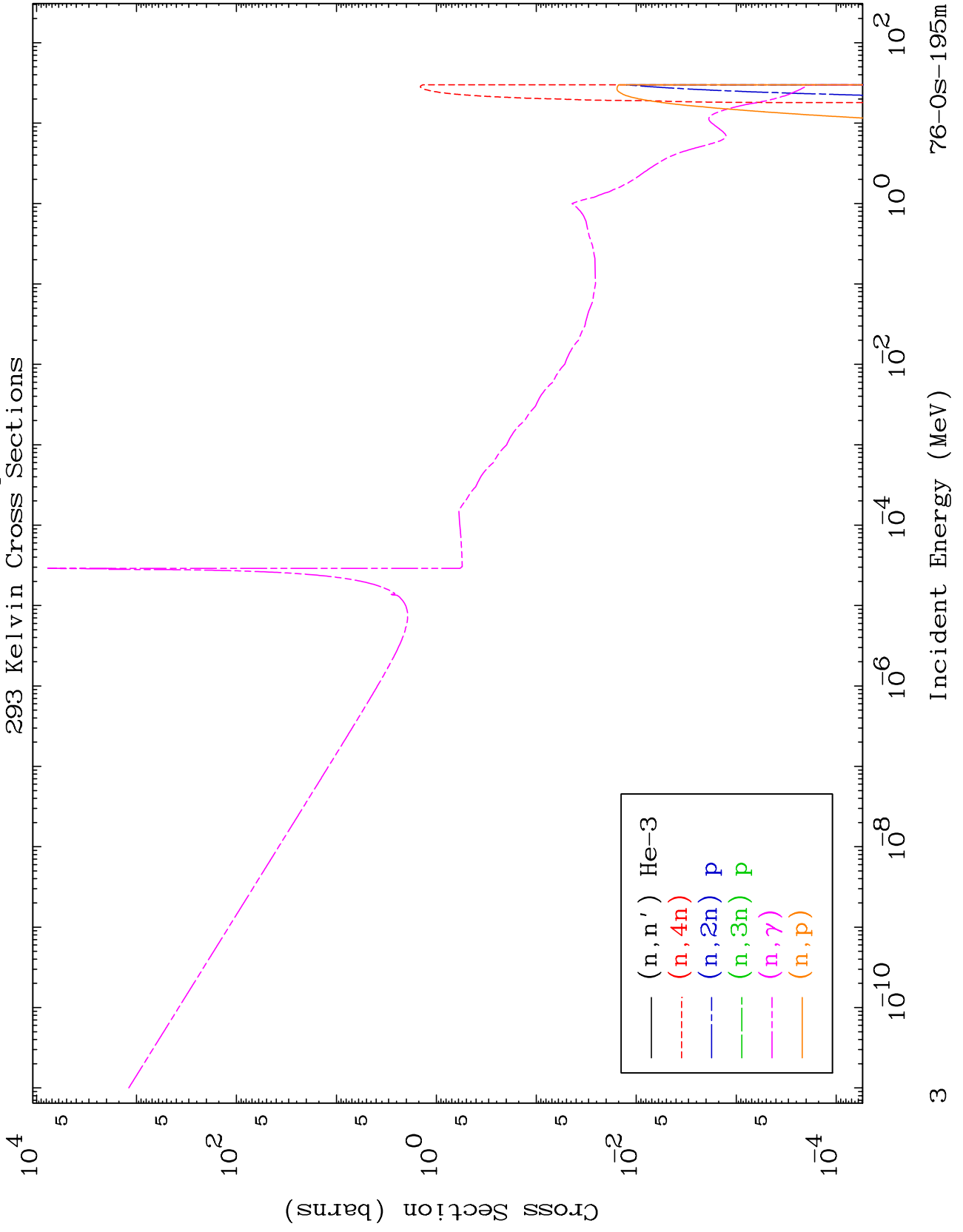
76-0s-195m



MAT 7659

Neutron Absorption
293 Kelvin Cross Sections

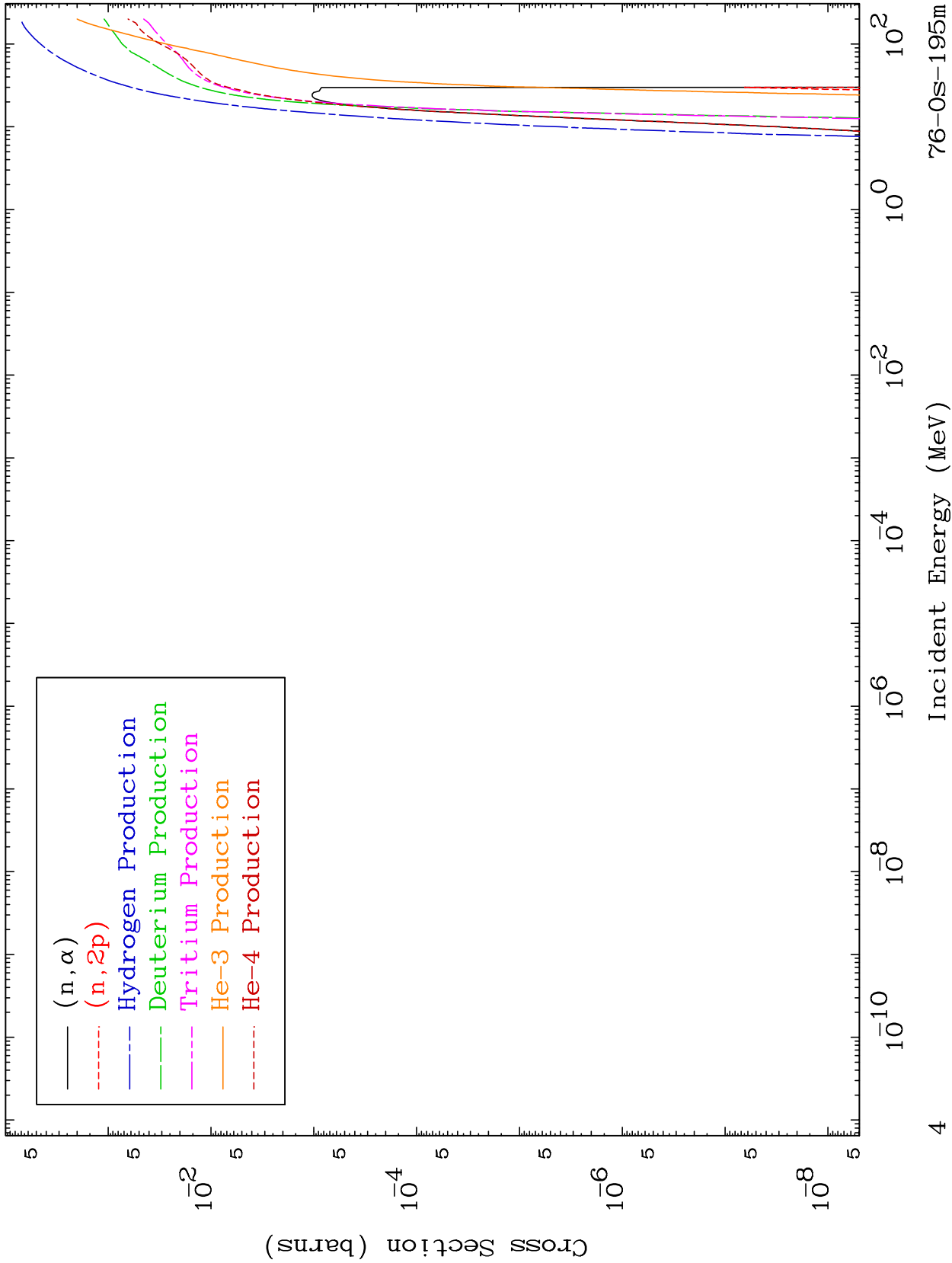
76-0s-195m



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Neutron Absorption
293 Kelvin Cross Sections

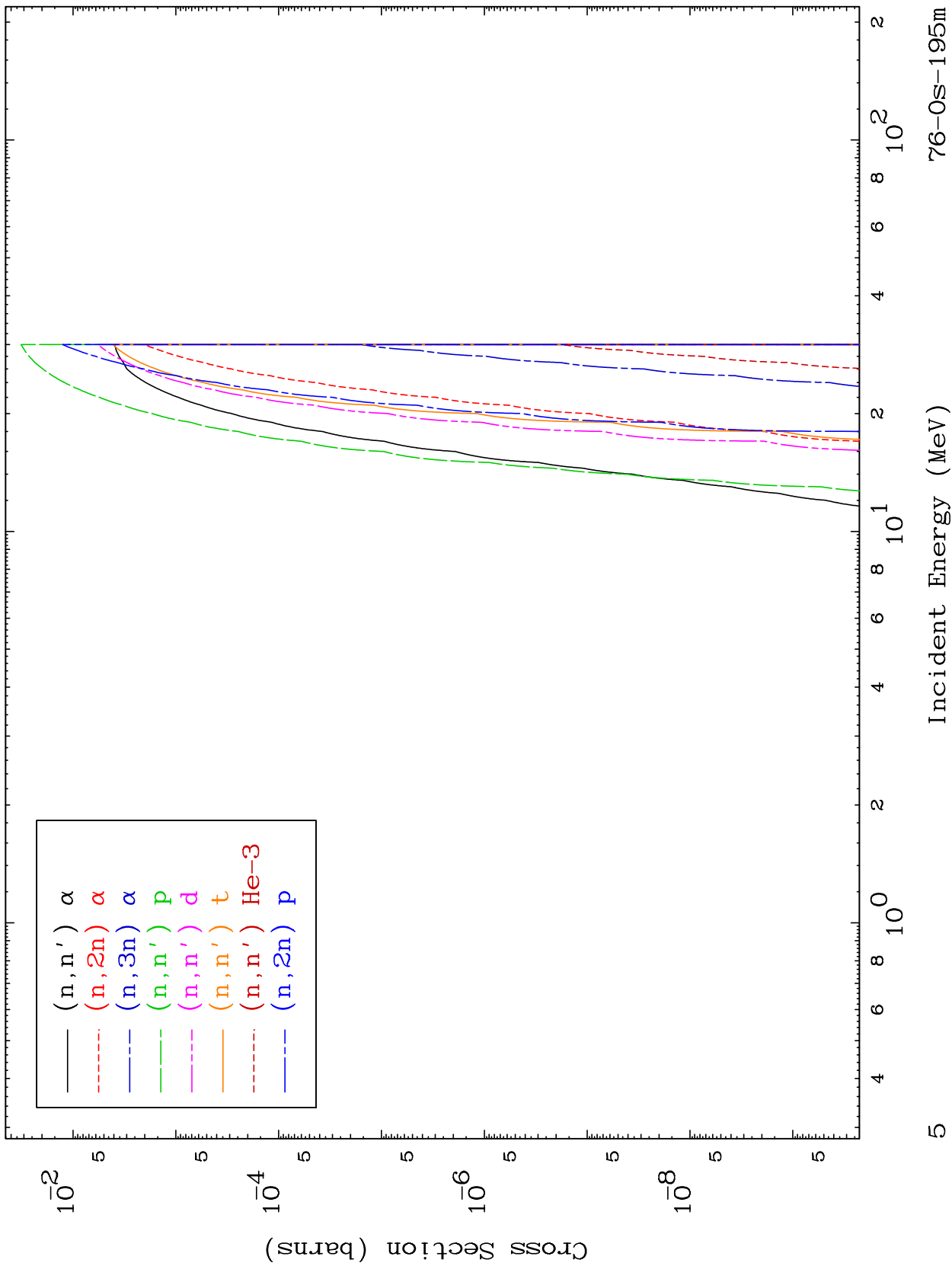
76-0s-195m



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Charged Particle
293 Kelvin Cross Sections

76-0s-195m

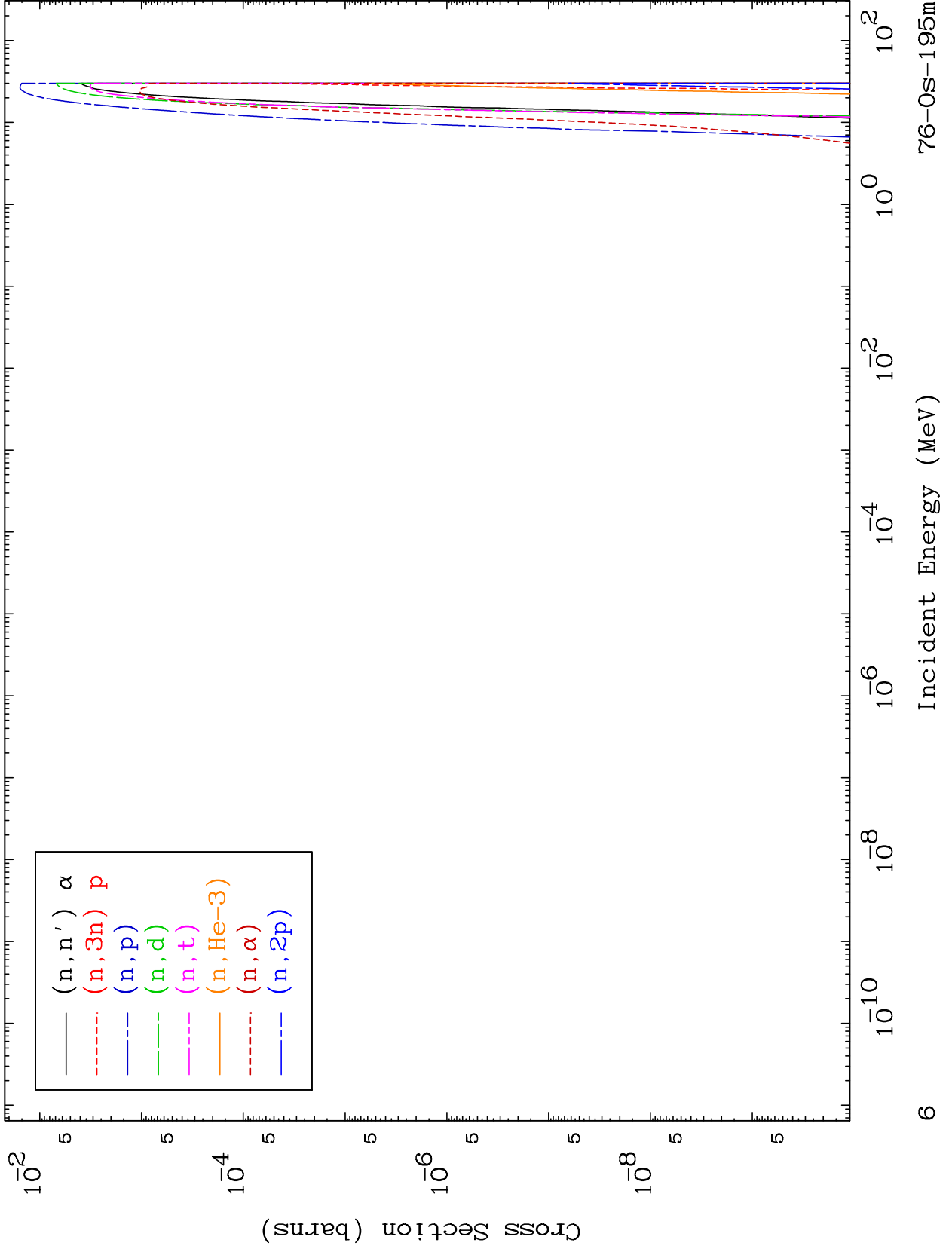


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Charged Particle
293 Kelvin Cross Sections

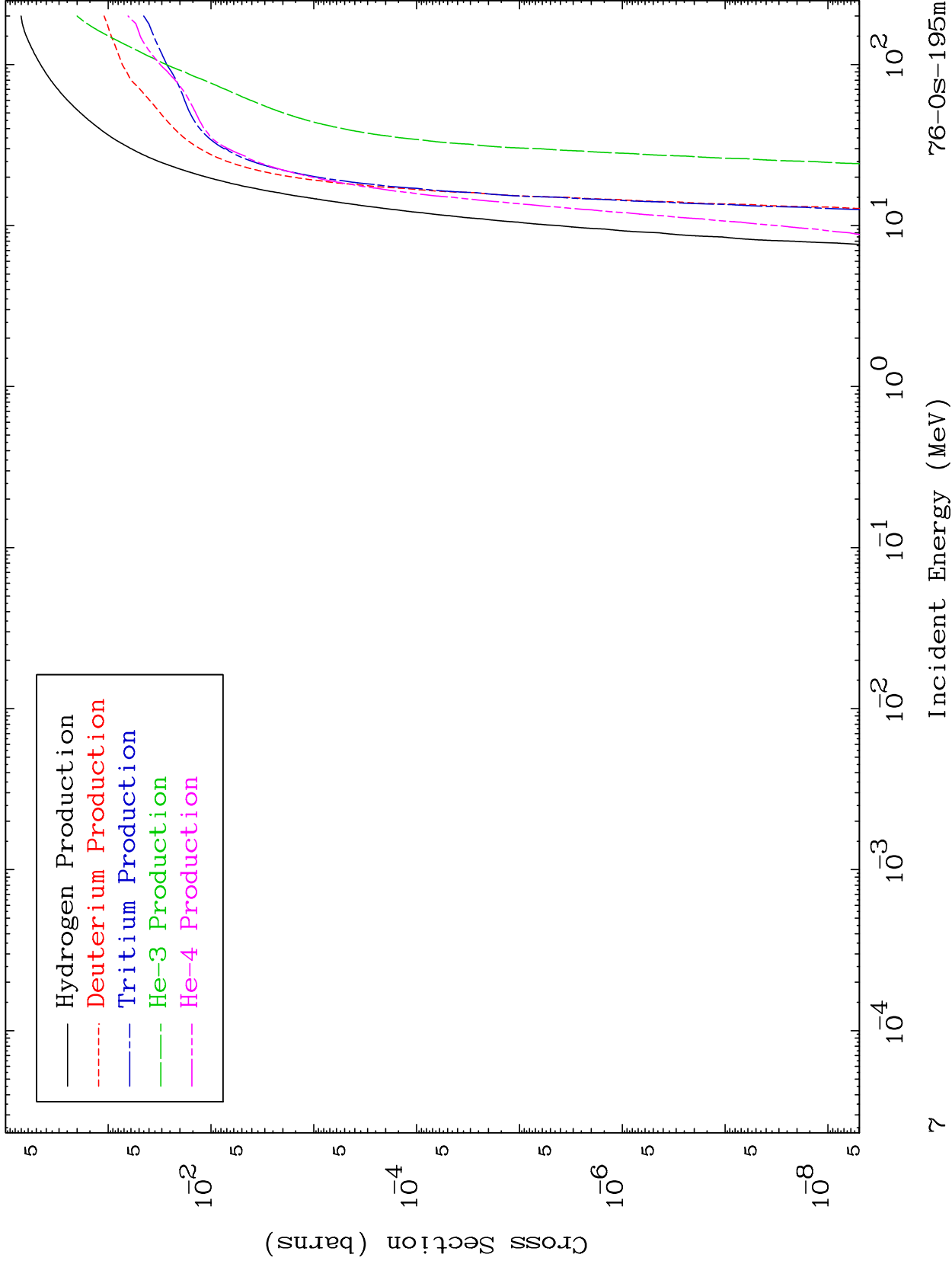
76-0s-195m



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Particle Production
293 Kelvin Cross Sections

76-0s-195m

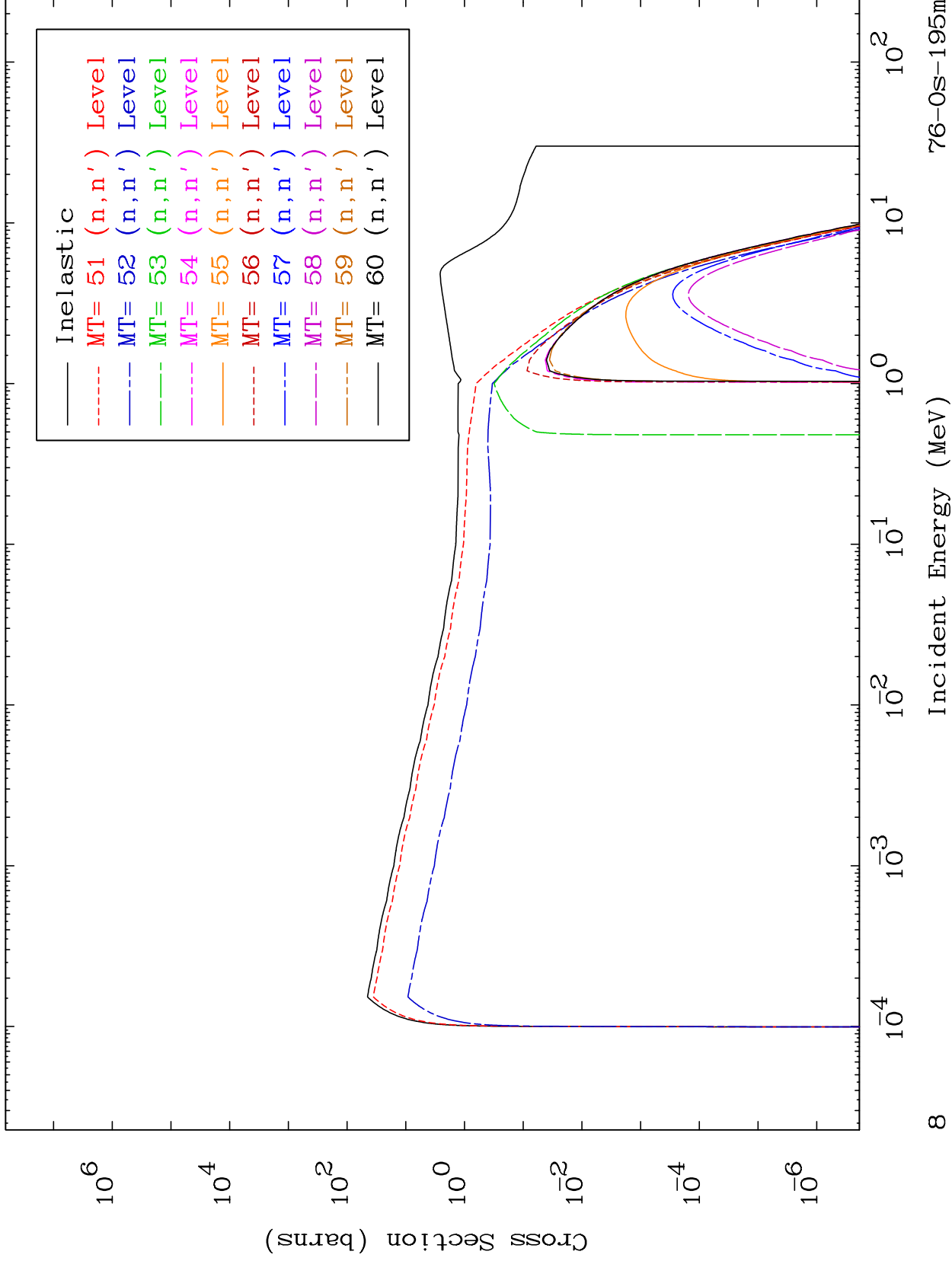


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(n,n') Levels

76-0s-195m

293 Kelvin Cross Sections

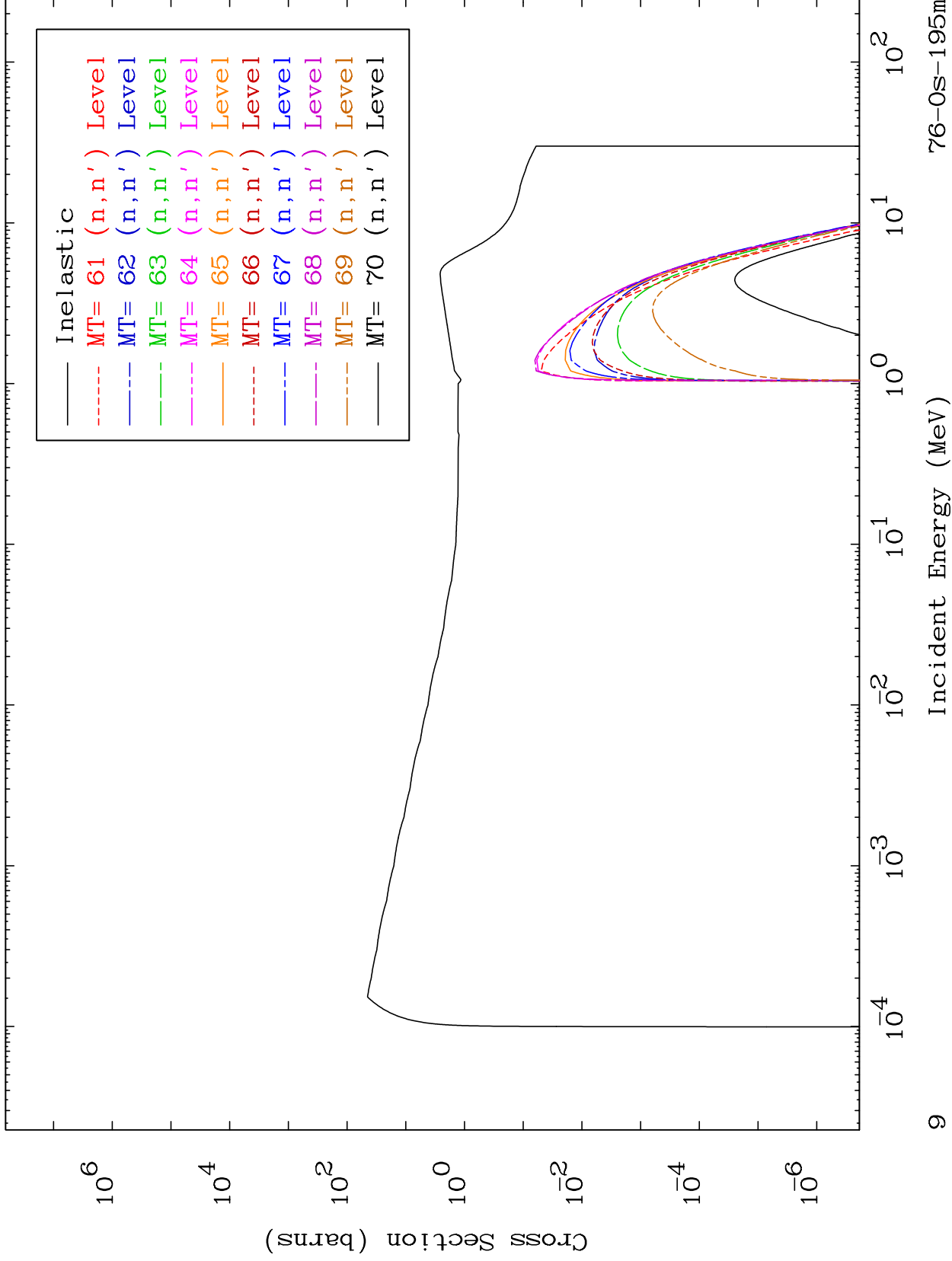


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(n,n') Levels

76-0s-195m

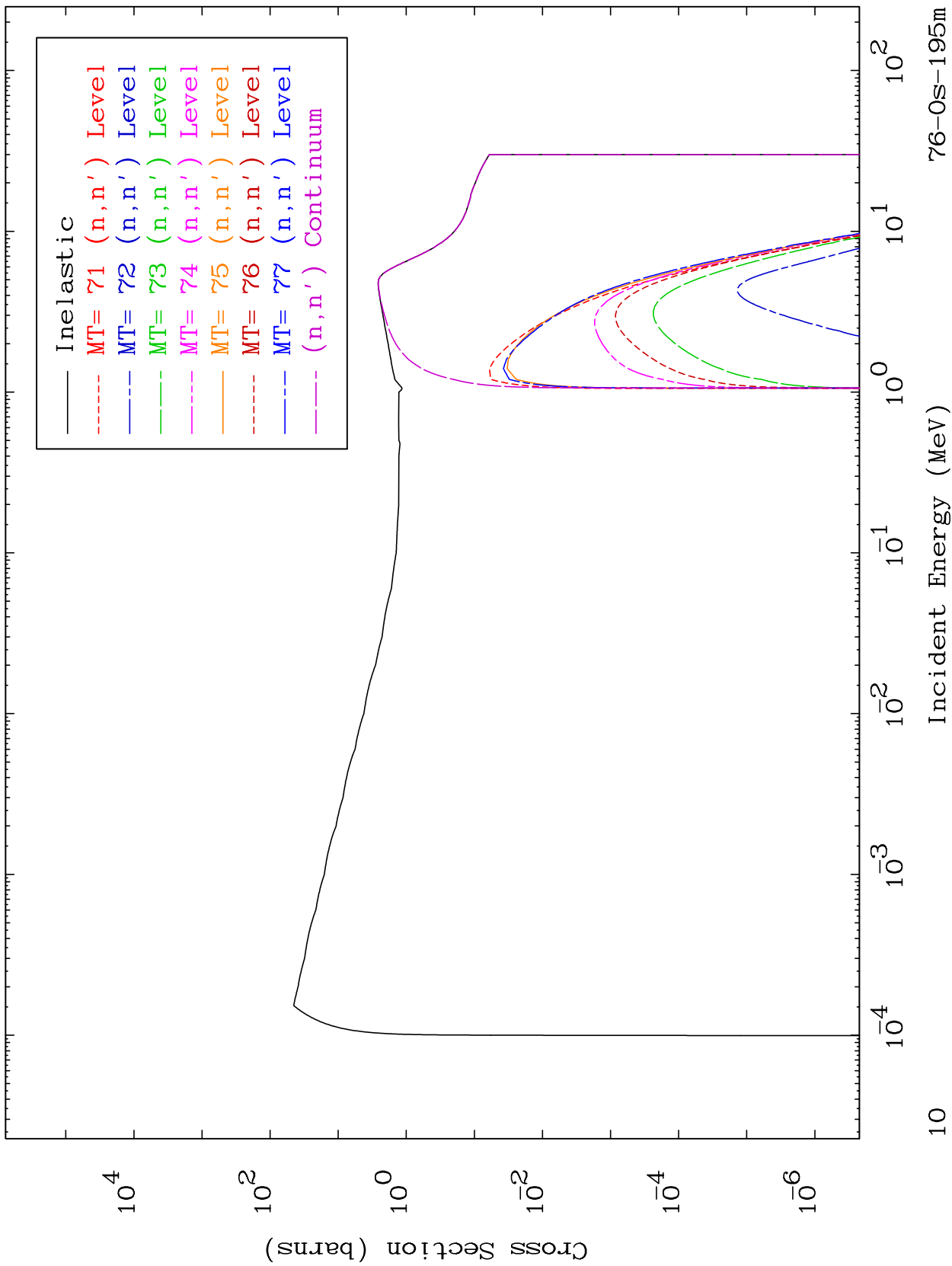
293 Kelvin Cross Sections



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(n,n') Levels
293 Kelvin Cross Sections

76-0s-195m



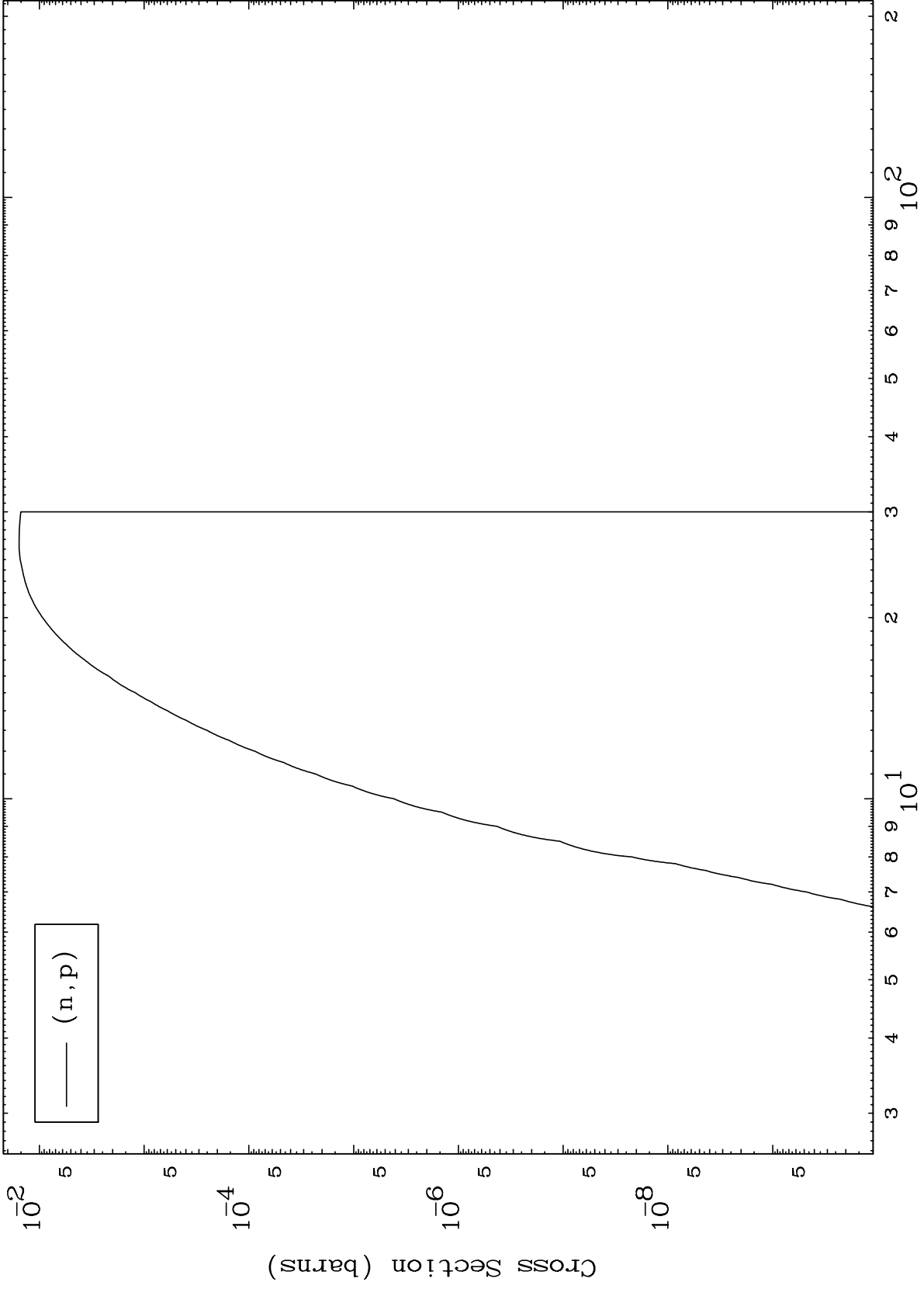
10

76-0s-195m

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(n,p) Levels
293 Kelvin Cross Sections

76-0s-195m



11

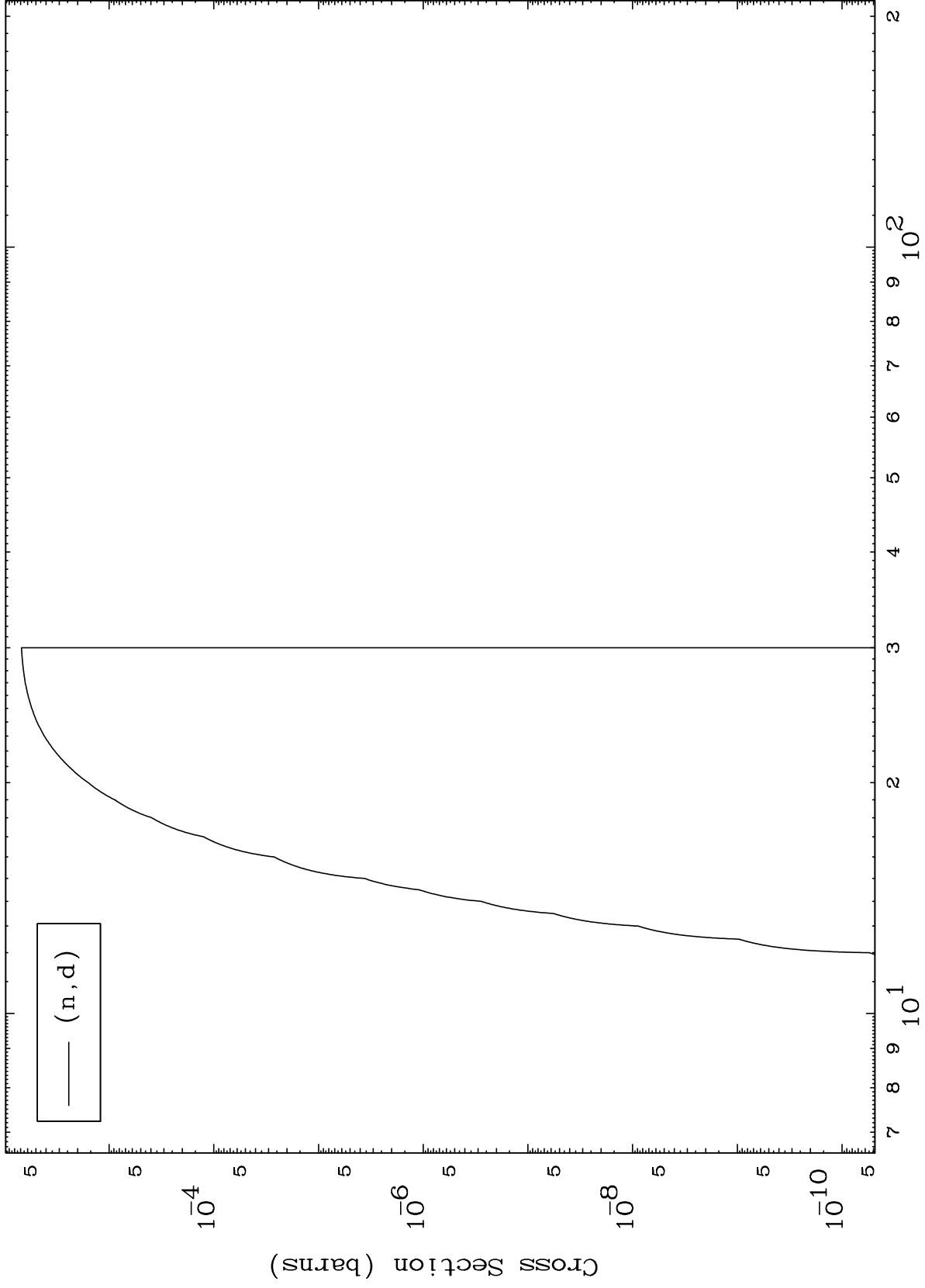
Incident Energy (MeV)

76-0s-195m

MAT 7659

(n,d) Levels
293 Kelvin Cross Sections

76-0s-195m



12

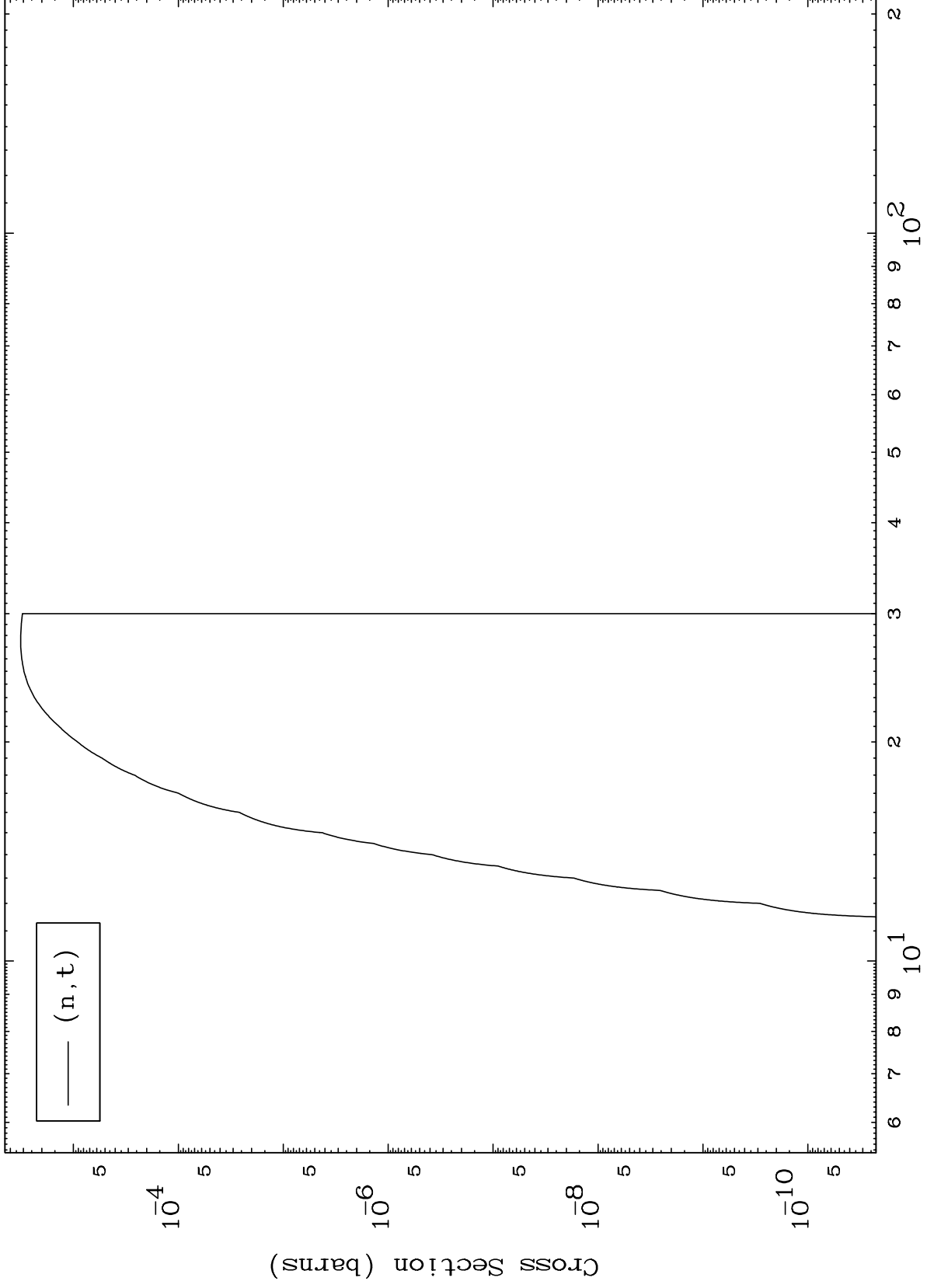
Incident Energy (MeV)

76-0s-195m

MAT 7659

(n,t) Levels
293 Kelvin Cross Sections

76-0s-195m



13

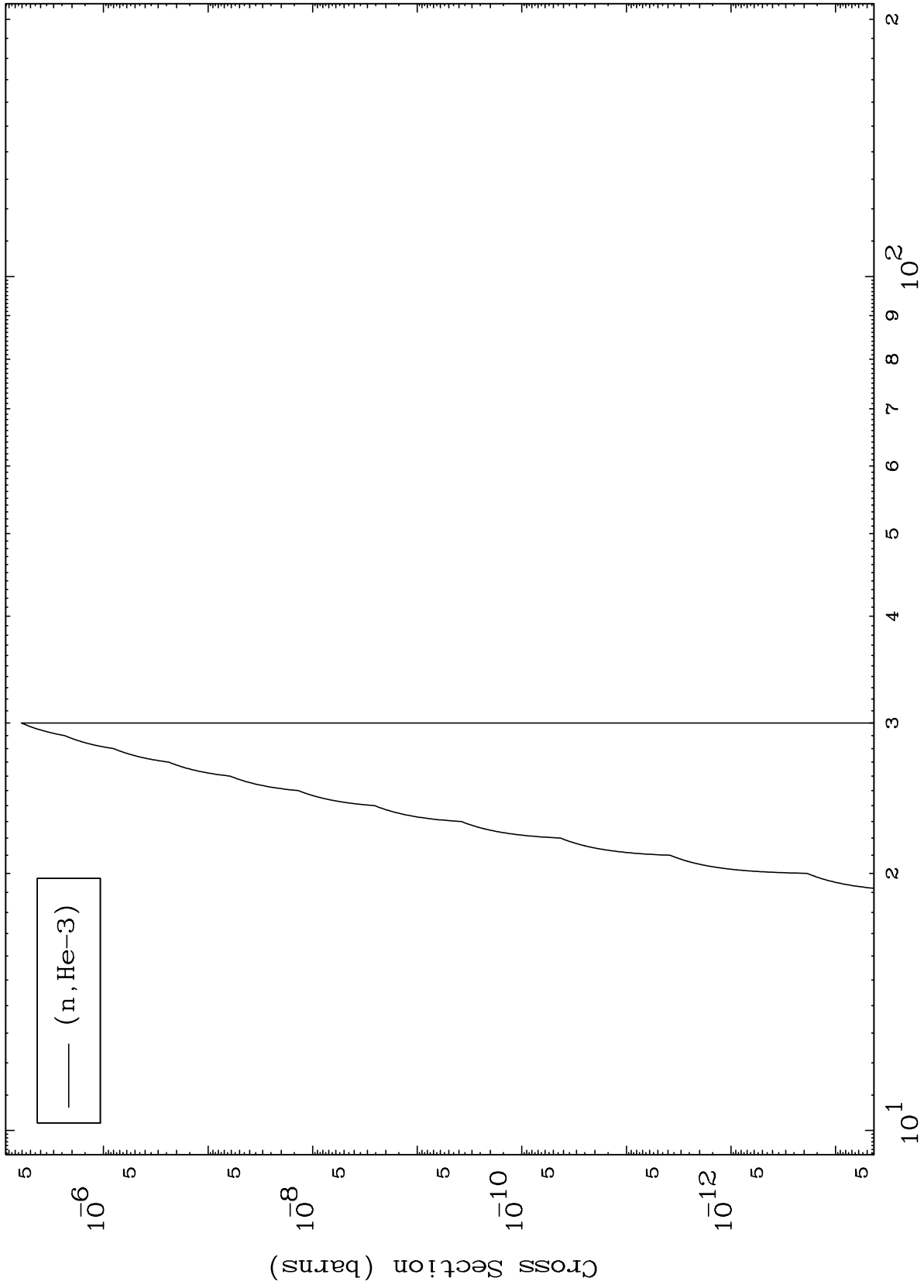
Incident Energy (MeV)

76-0s-195m

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(n,He3) Levels
293 Kelvin Cross Sections

76-0s-195m



14

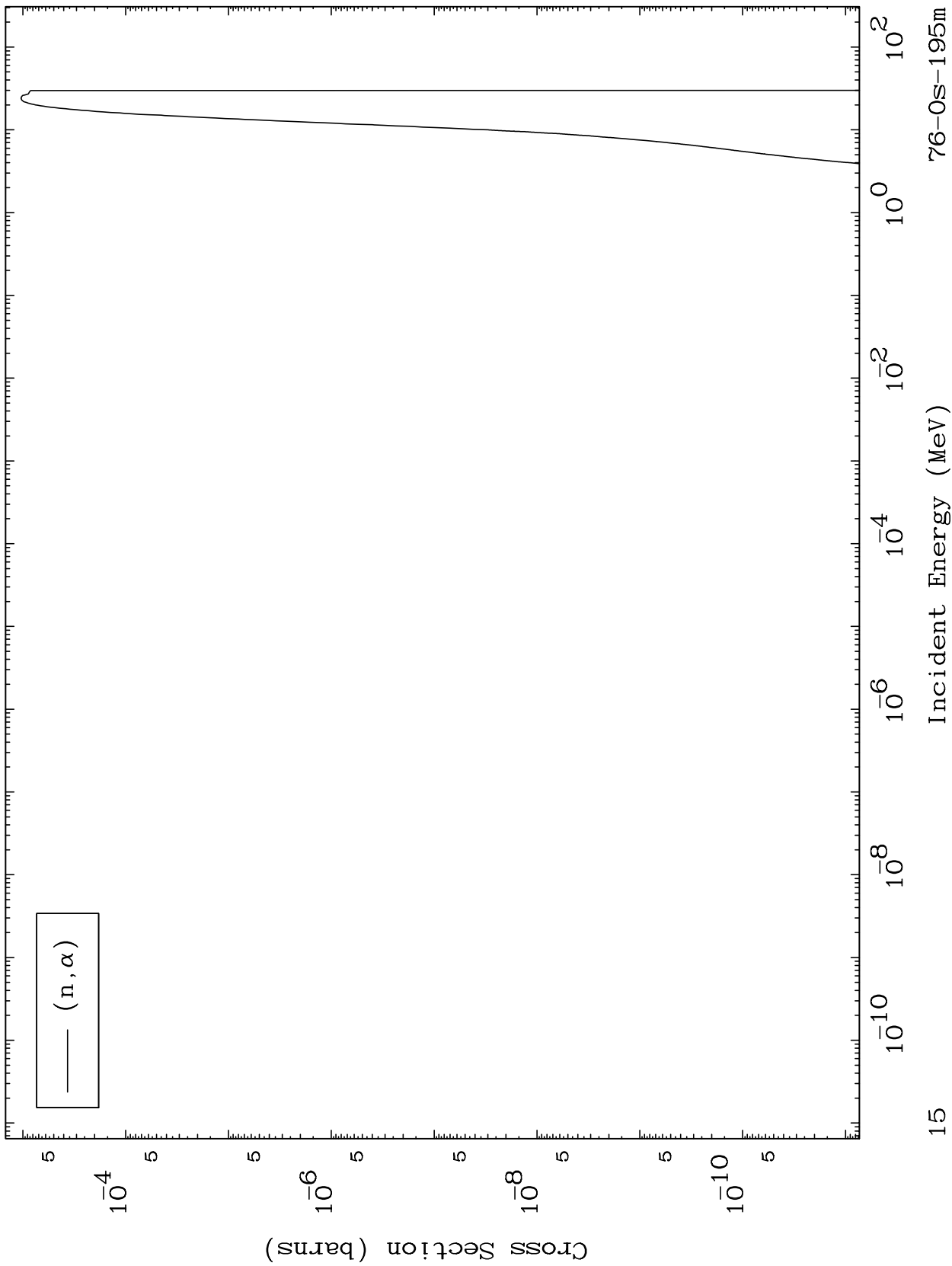
Incident Energy (MeV)

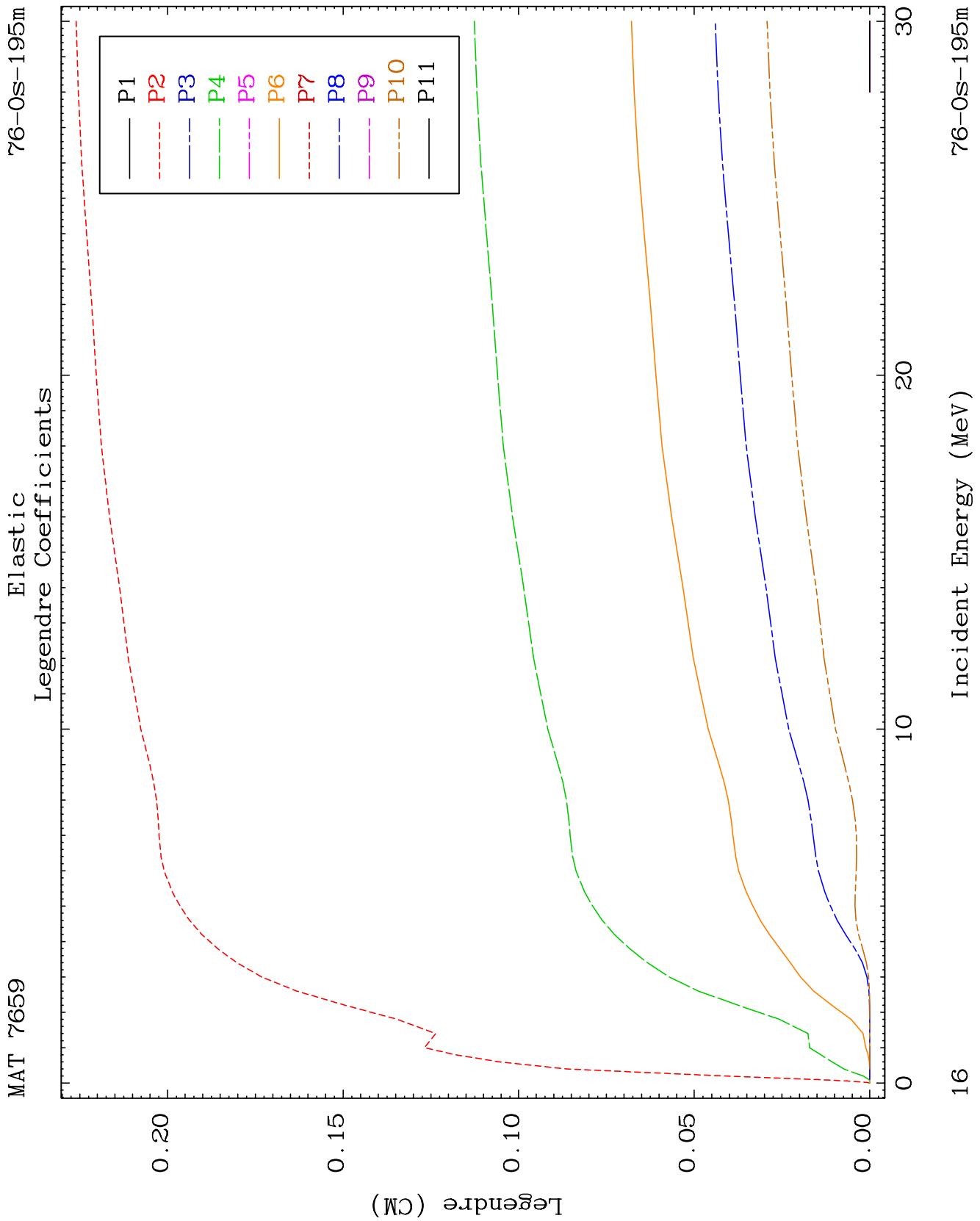
76-0s-195m

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(n, α) Levels
293 Kelvin Cross Sections

76-0s-195m

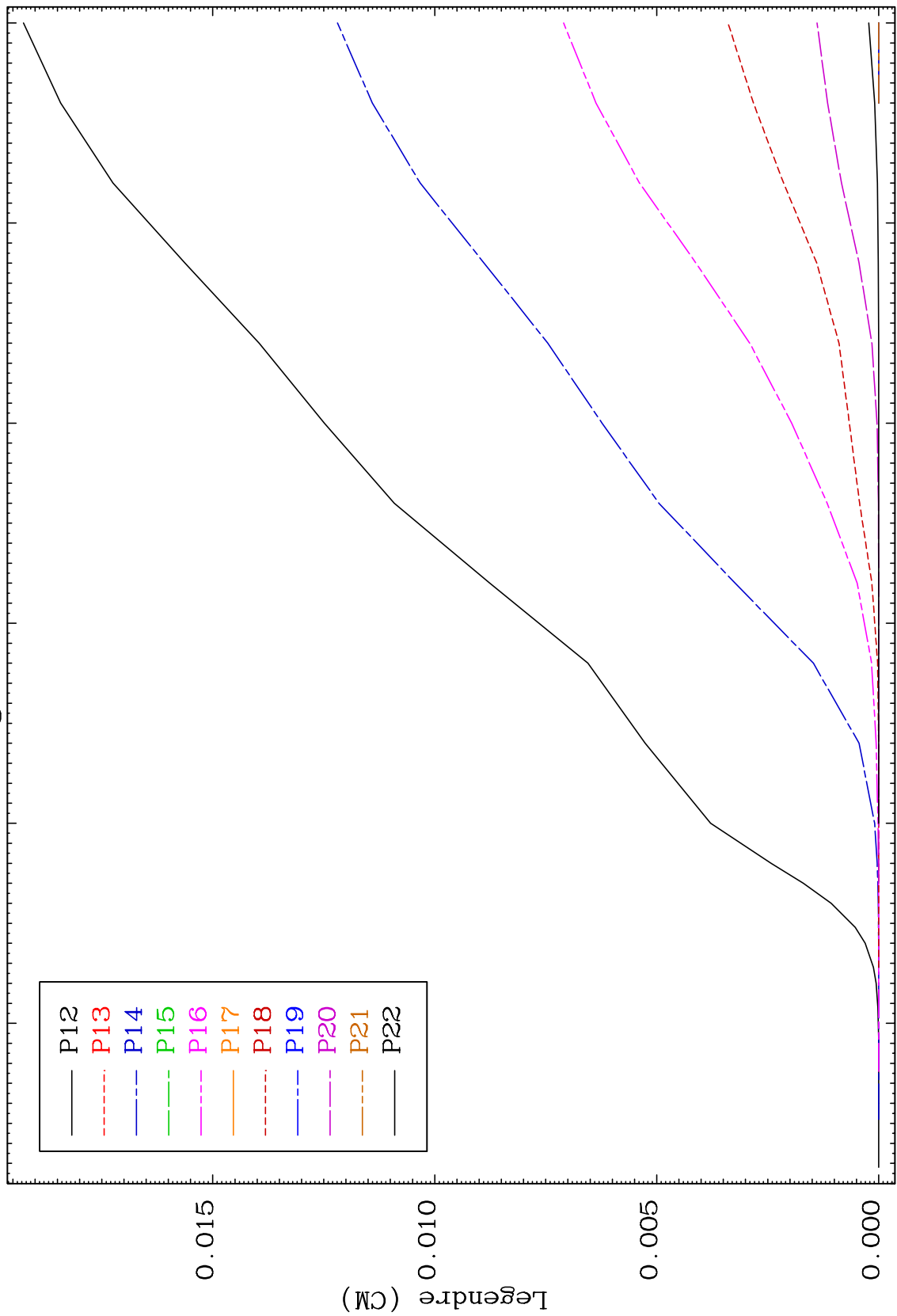




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Elastic Legendre Coefficients

76-0s-195m



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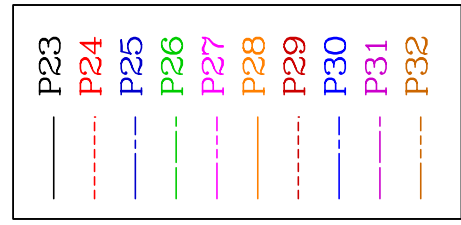
Incident Energy (MeV)

76-0s-195m

MAT 7659

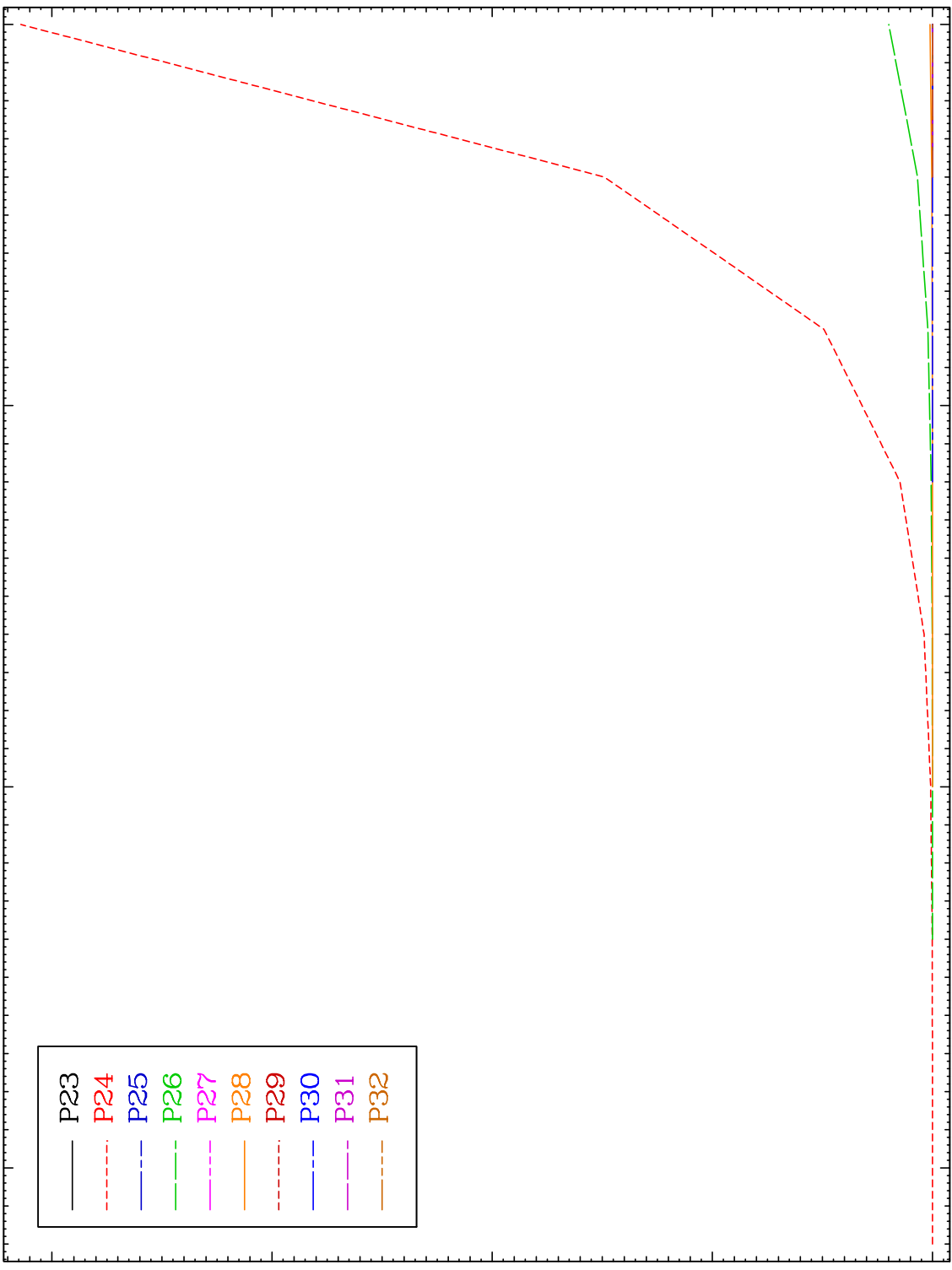
Elastic Legendre Coefficients

76-0s-195m



$\times 10^{-68}$

Legendre (CM)



30

25

20

15

18

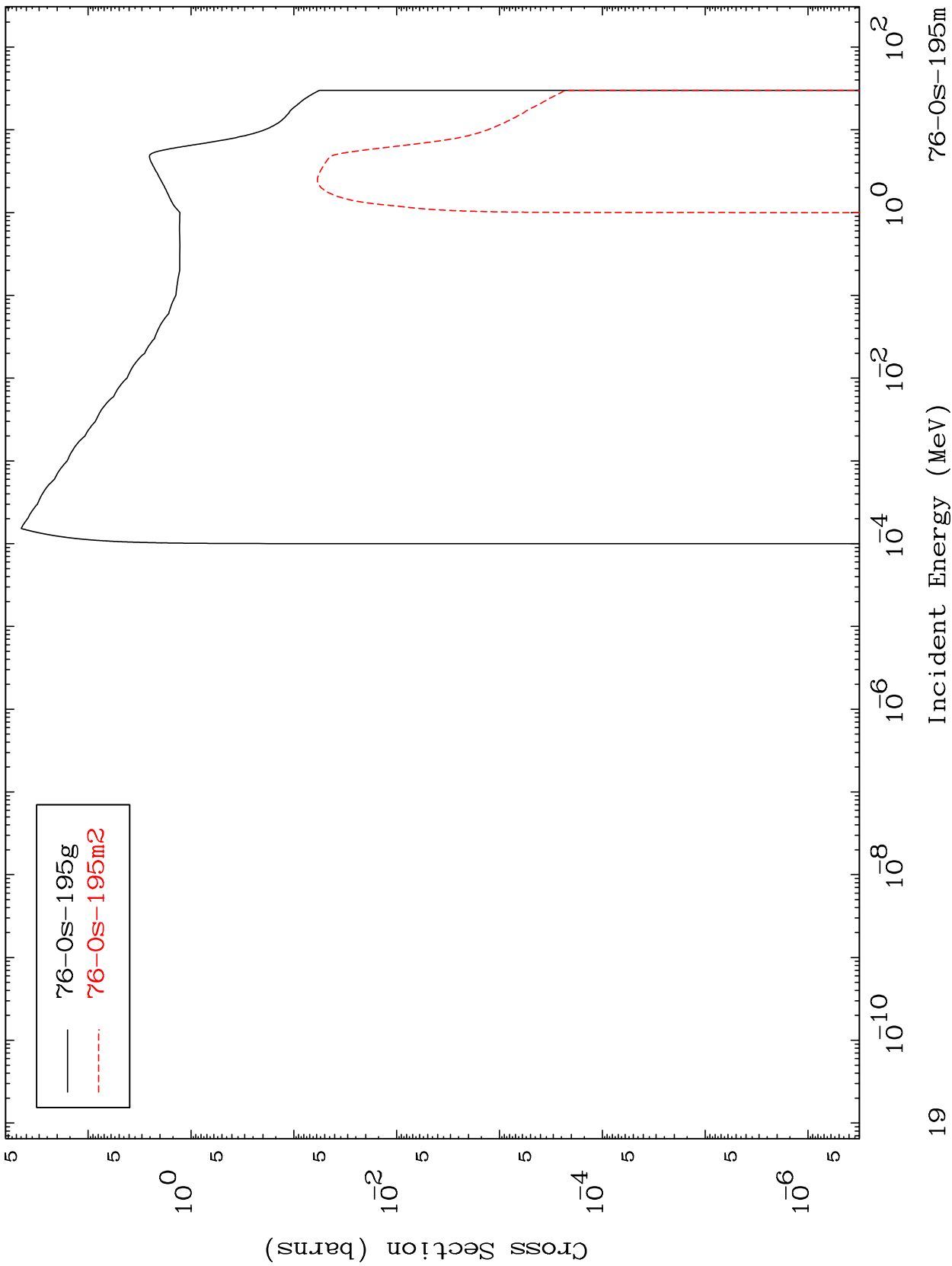
Incident Energy (MeV)

76-0s-195m

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76-0s-195m

Inelastic
Radionuclide Production Cross Section

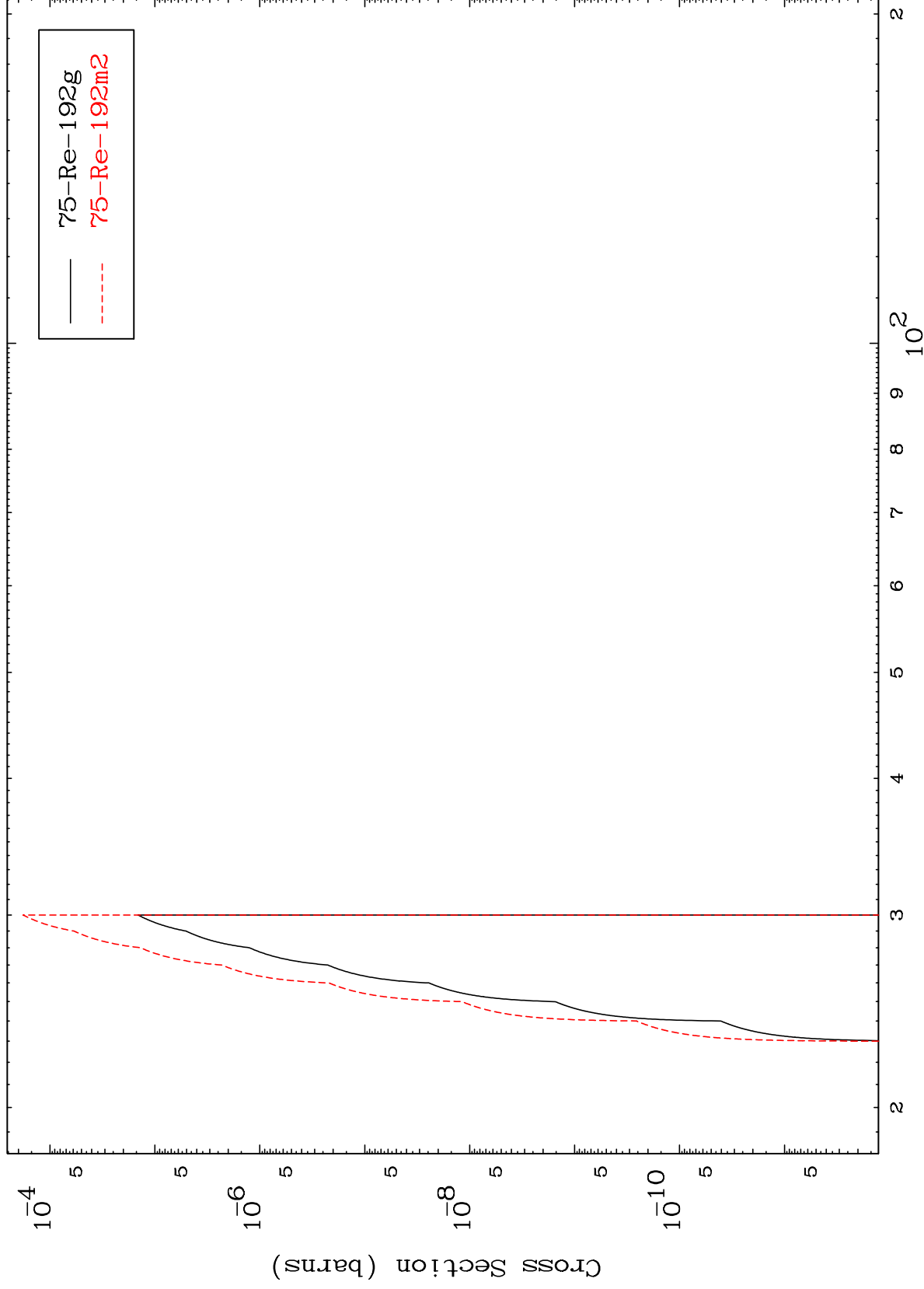


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(n,2n) d

76-0s-195m

Radionuclide Production Cross Section



20

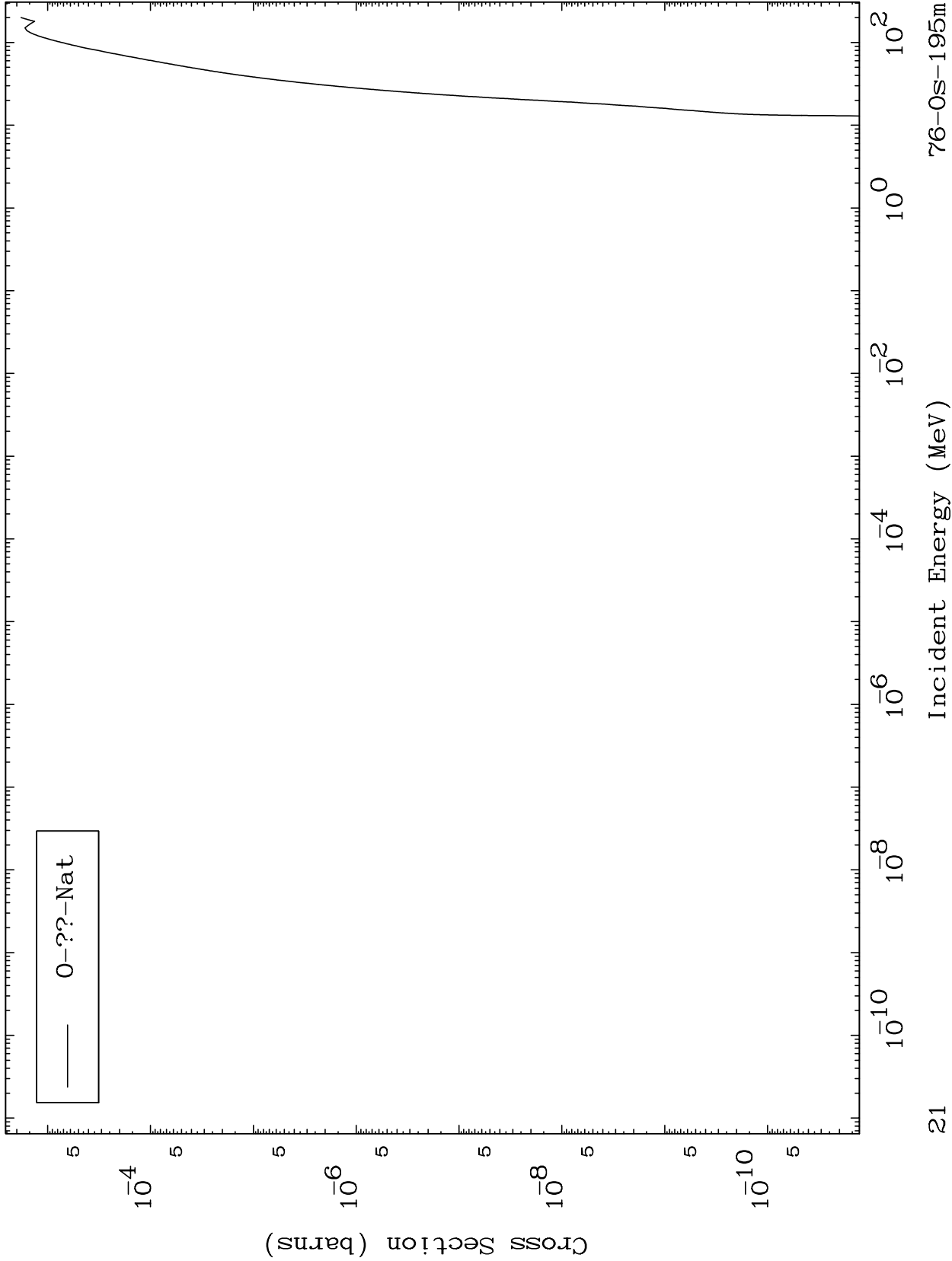
Incident Energy (MeV)

76-0s-195m

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Fission
Radionuclide Production Cross Section

76-0s-195m

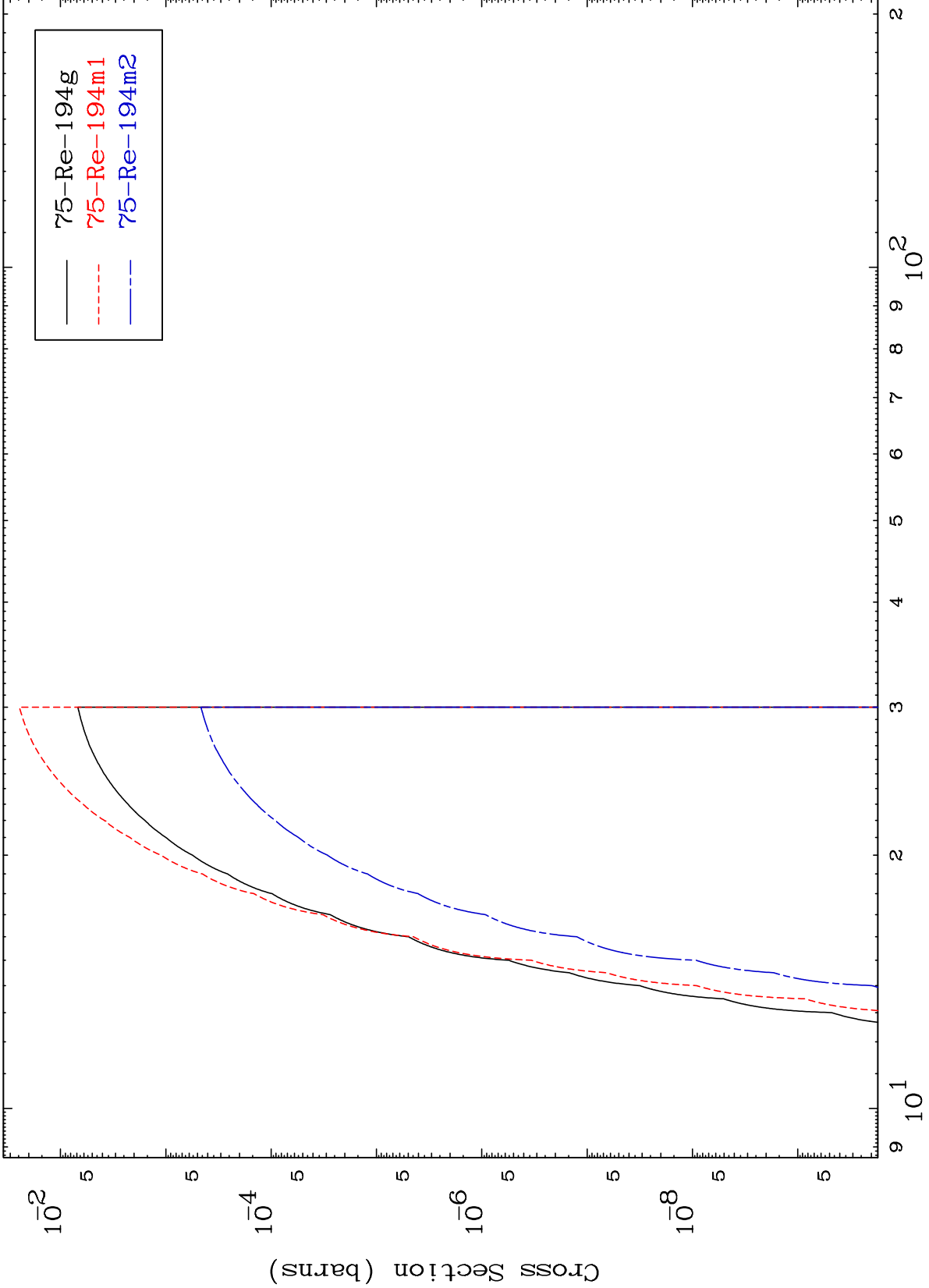


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(n,n') p

76-0s-195m

Radionuclide Production Cross Section



22

Incident Energy (MeV)

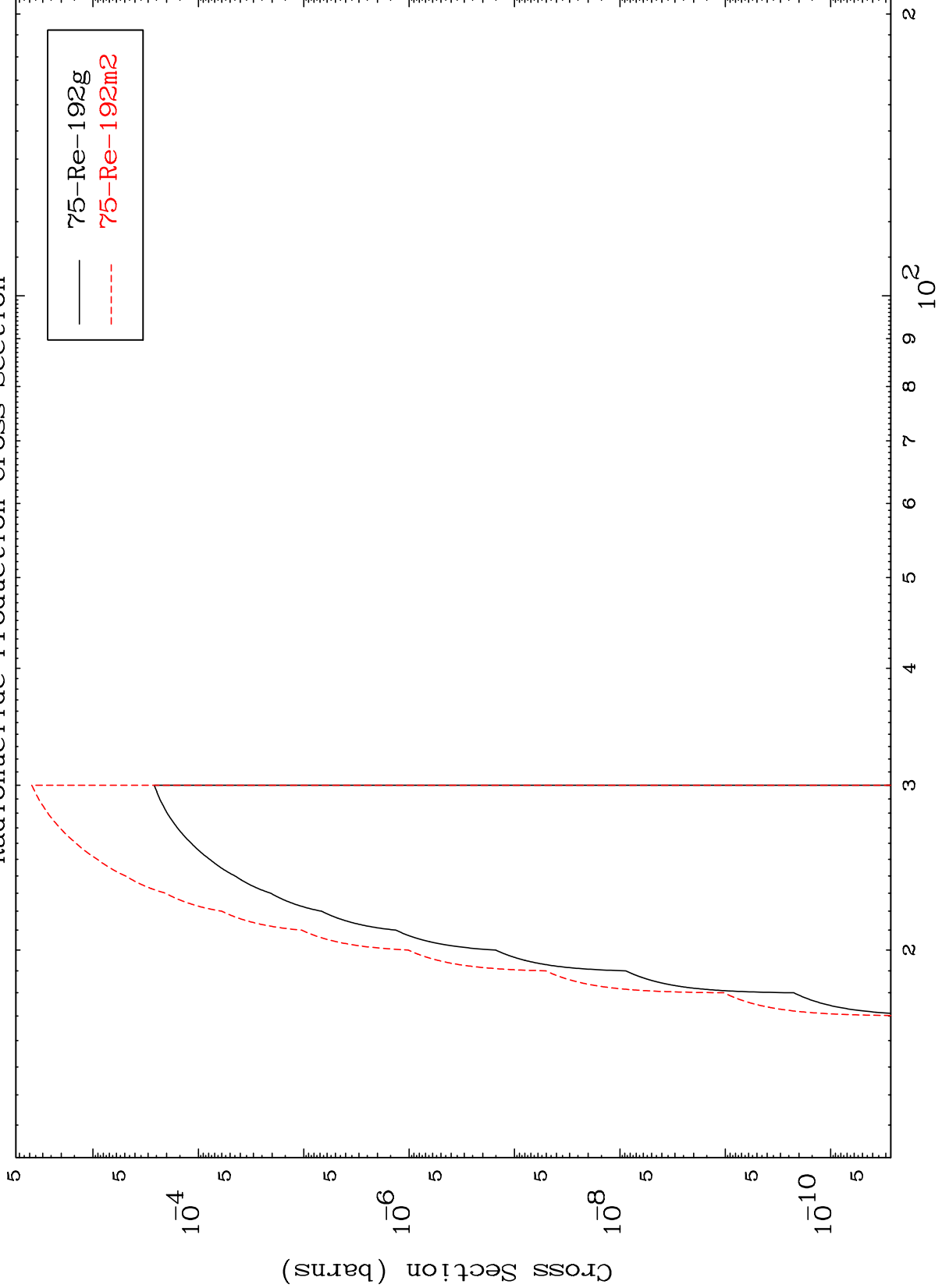
76-0s-195m

MAT 7659

(n,n') t

76-0s-195m

Radionuclide Production Cross Section

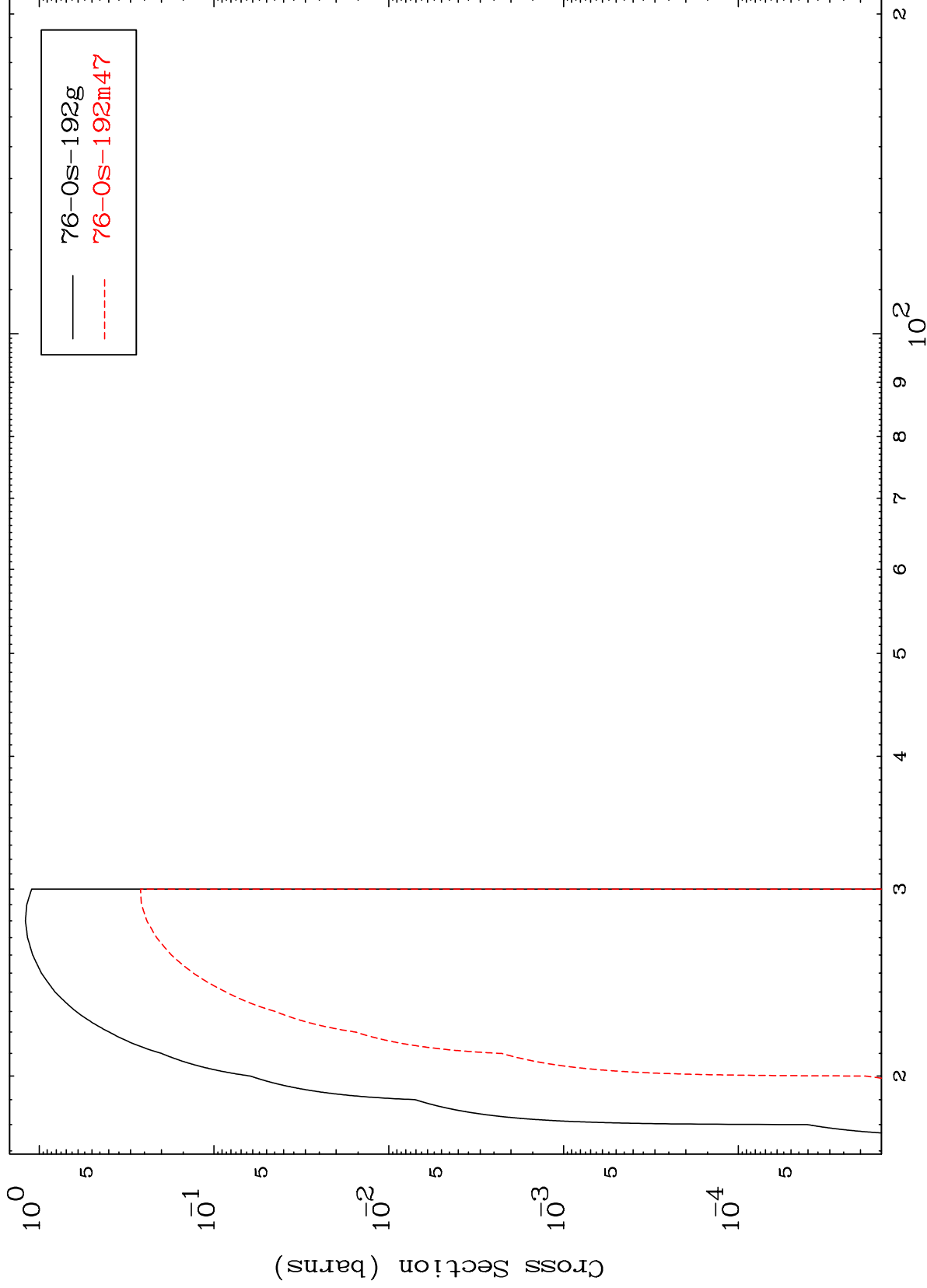


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(n,4n)

76-0s-195m

Radionuclide Production Cross Section



24

Incident Energy (MeV)

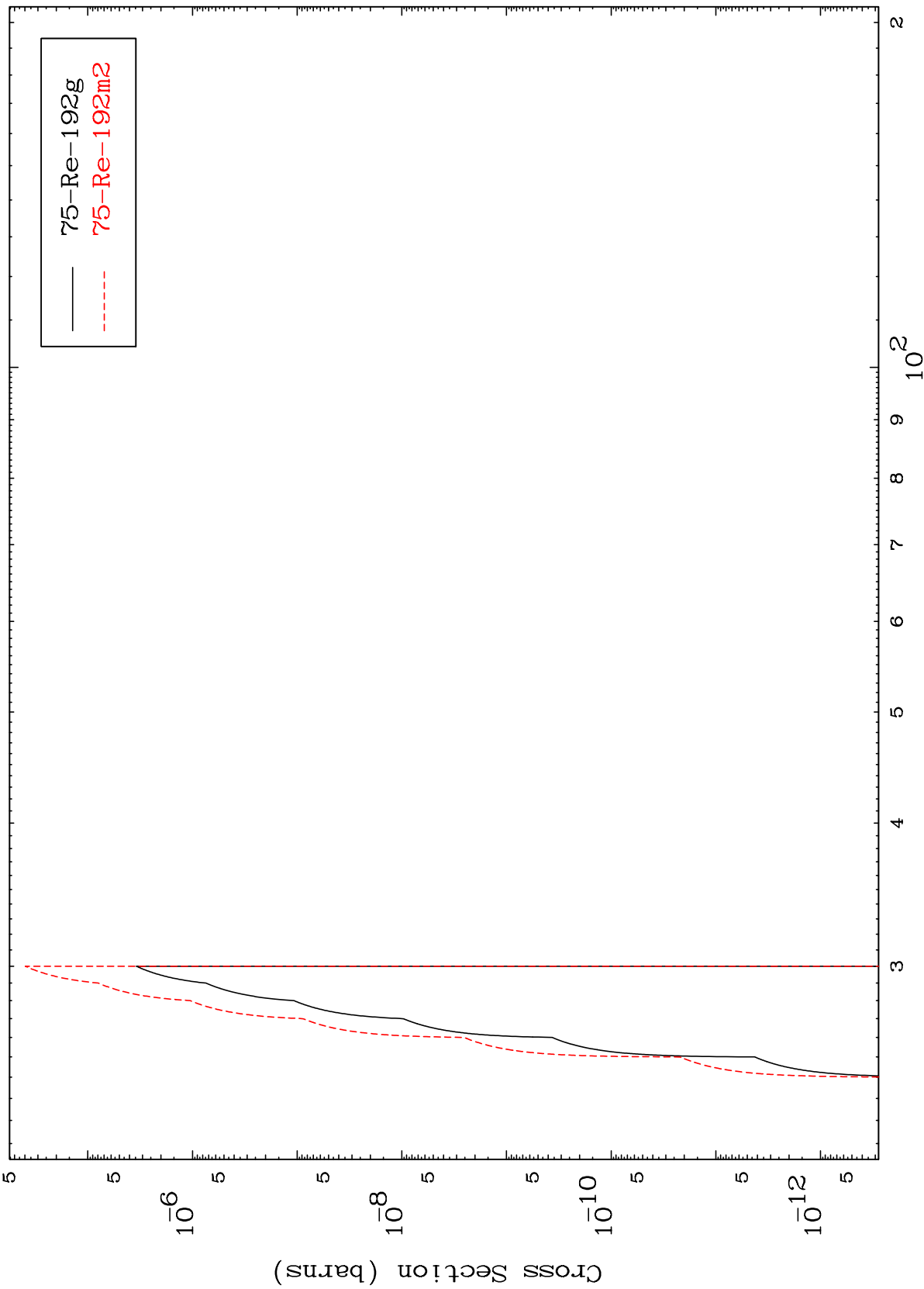
76-0s-195m

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(n,3n) p

76-0s-195m

Radionuclide Production Cross Section



25

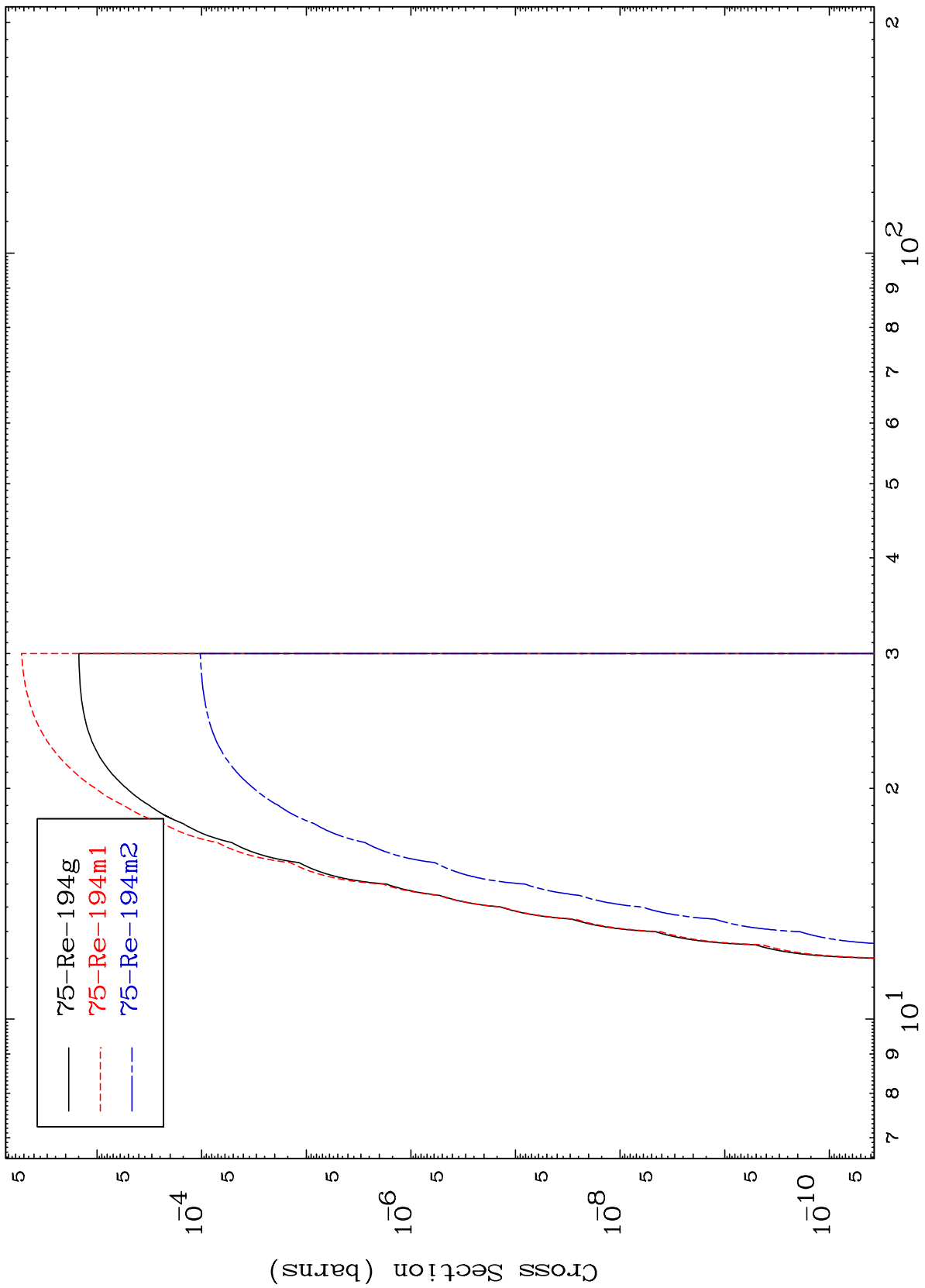
Incident Energy (MeV)

76-0s-195m

MAT 7659

76-0s-195m

(n,d)
Radionuclide Production Cross Section



26

Incident Energy (MeV)

76-0s-195m