

Program EVALPLOT  
(Version 2021-1)

by

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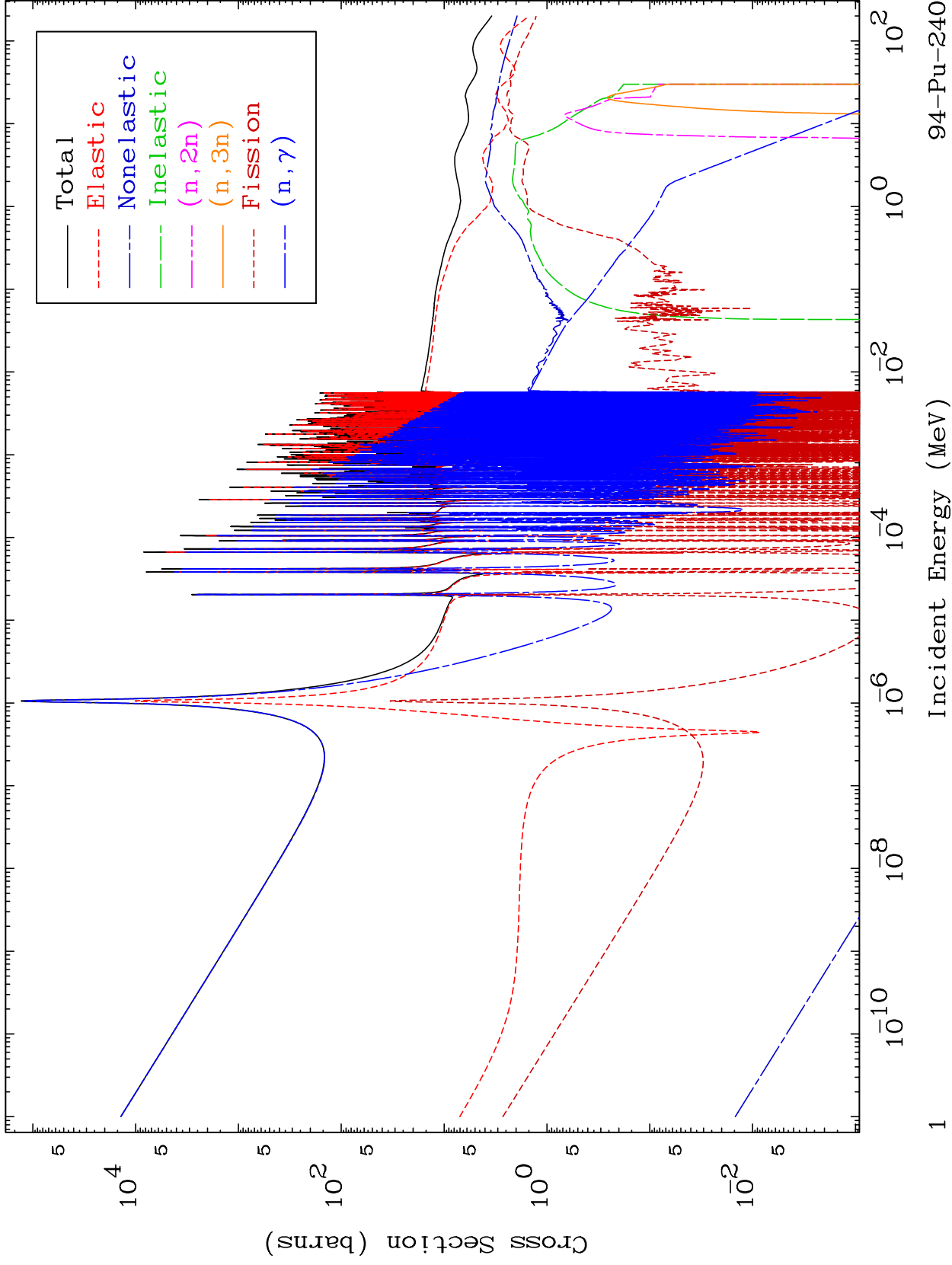
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9440

Neutron Major  
293 Kelvin Cross Sections

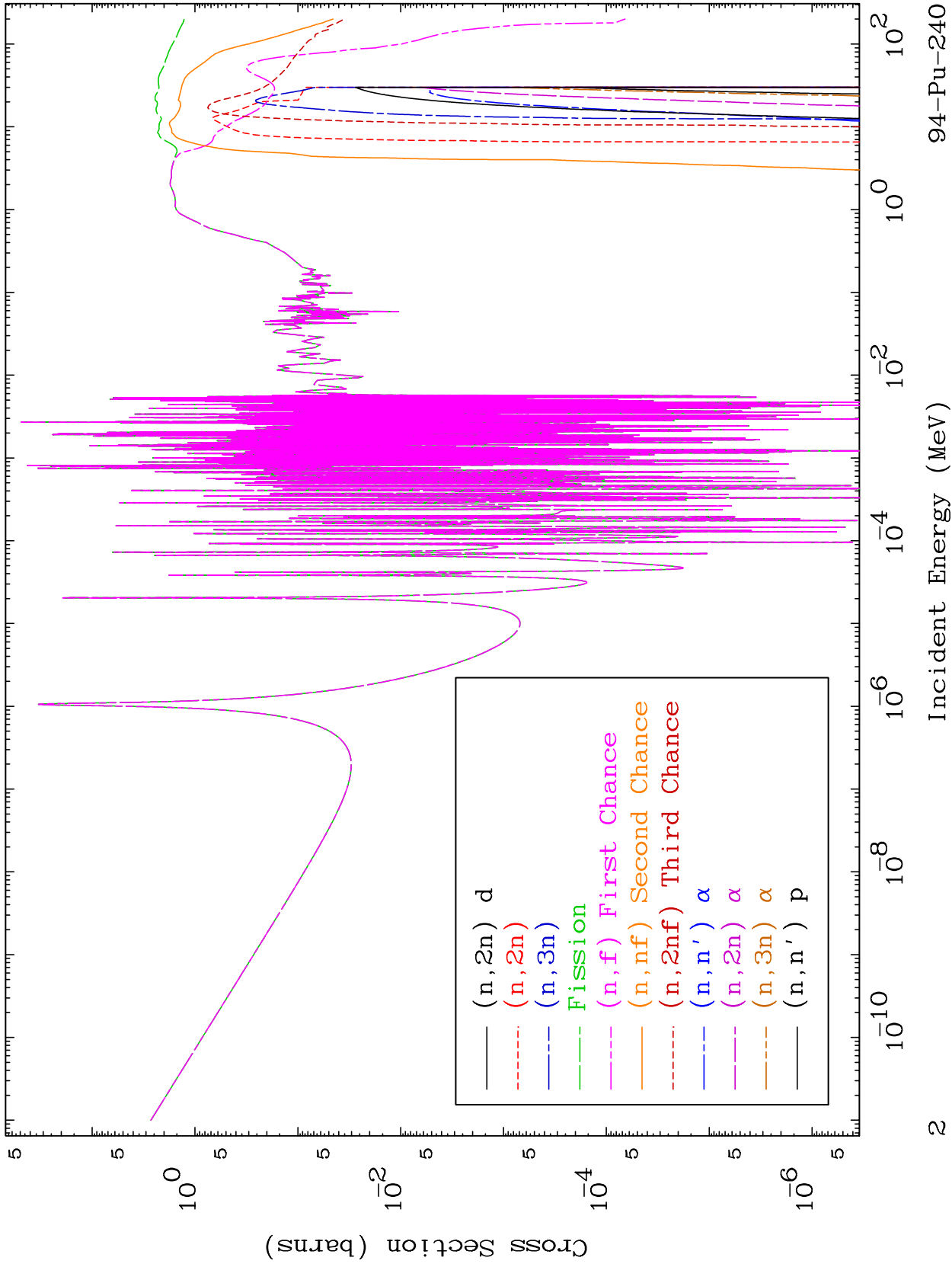
94-Pu-240



MAT 9440

Neutron Absorption  
293 Kelvin Cross Sections

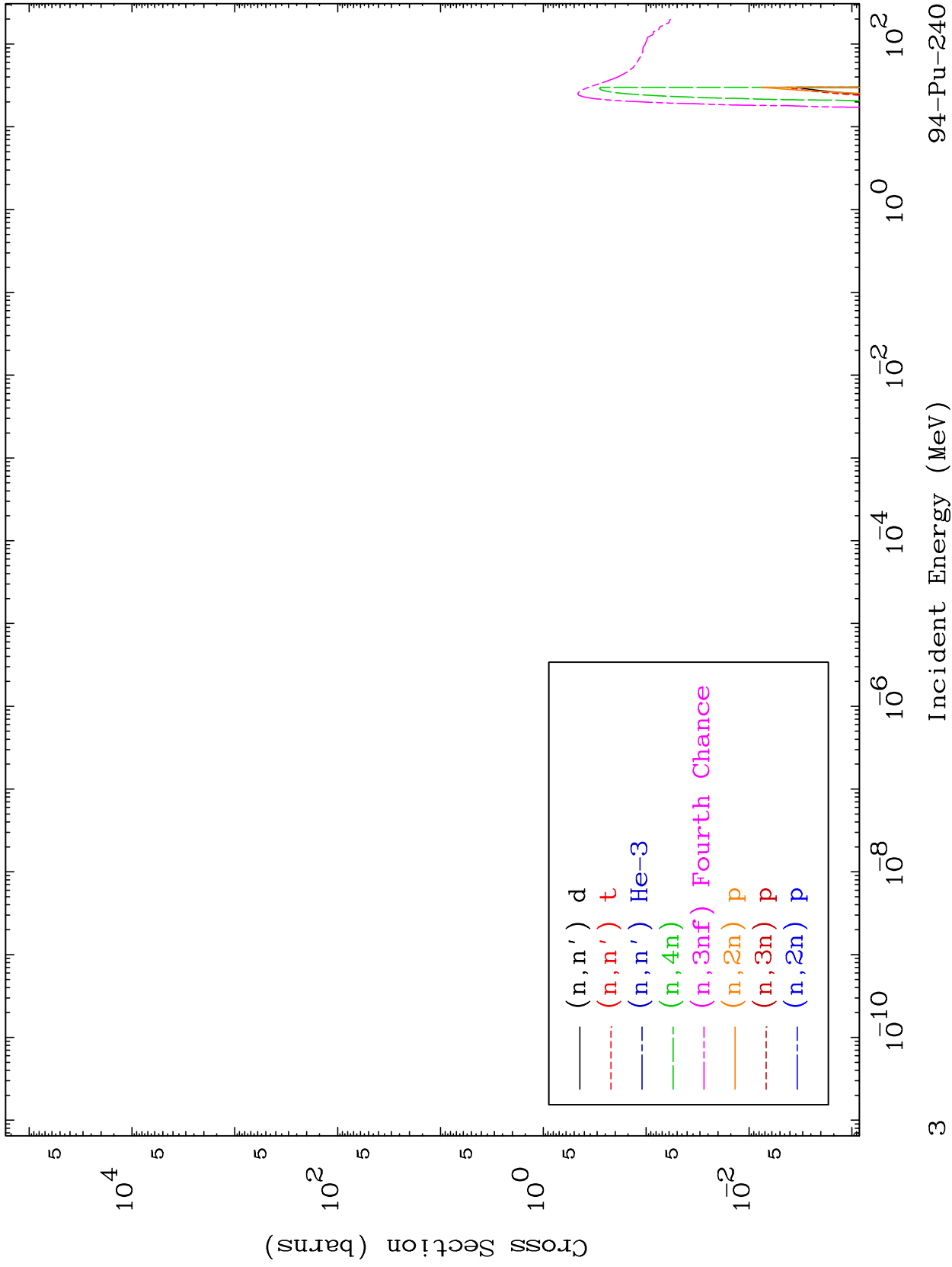
94-Pu-240



MAT 9440

Neutron Absorption  
293 Kelvin Cross Sections

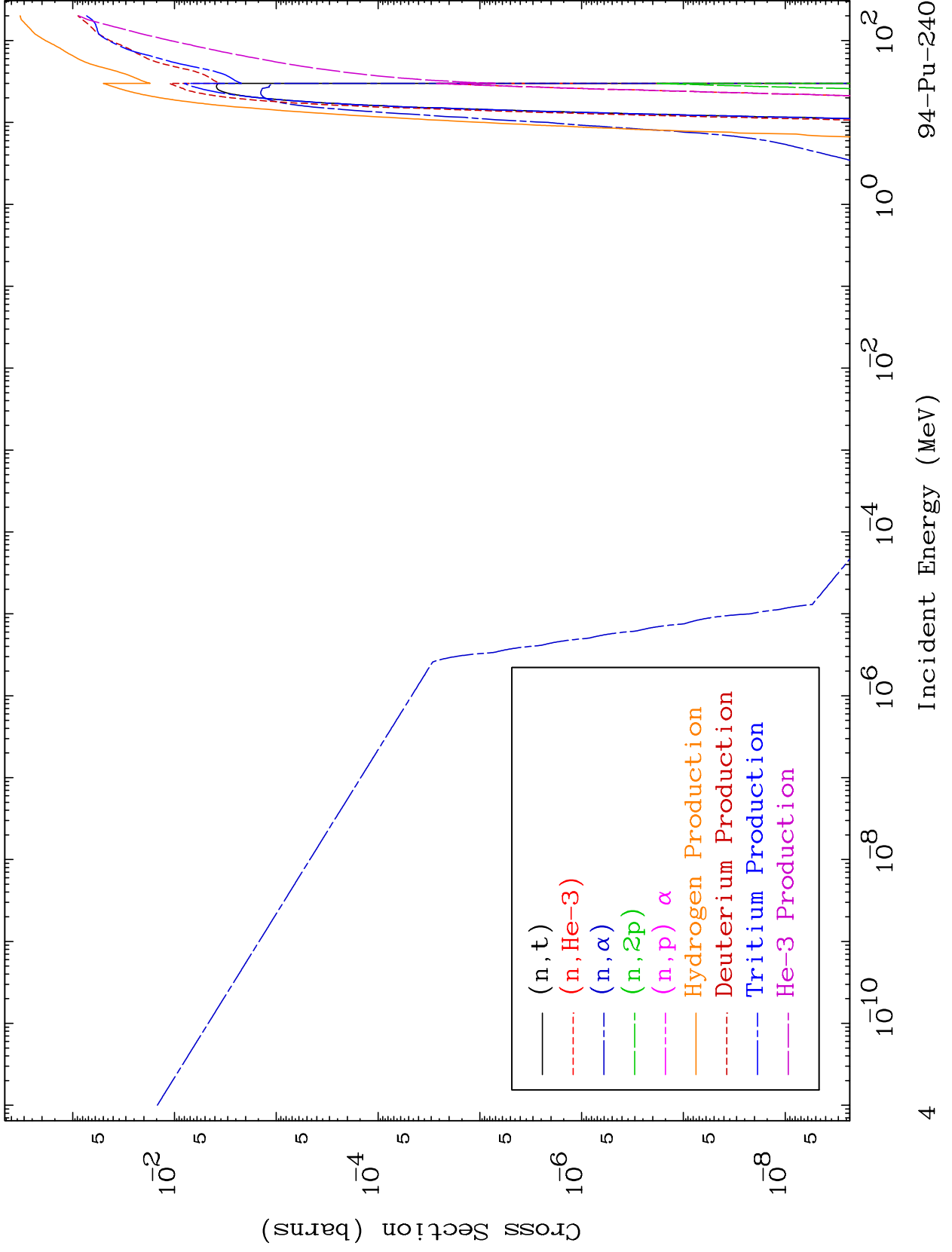
94-Pu-240



MAT 9440

Neutron Absorption  
293 Kelvin Cross Sections

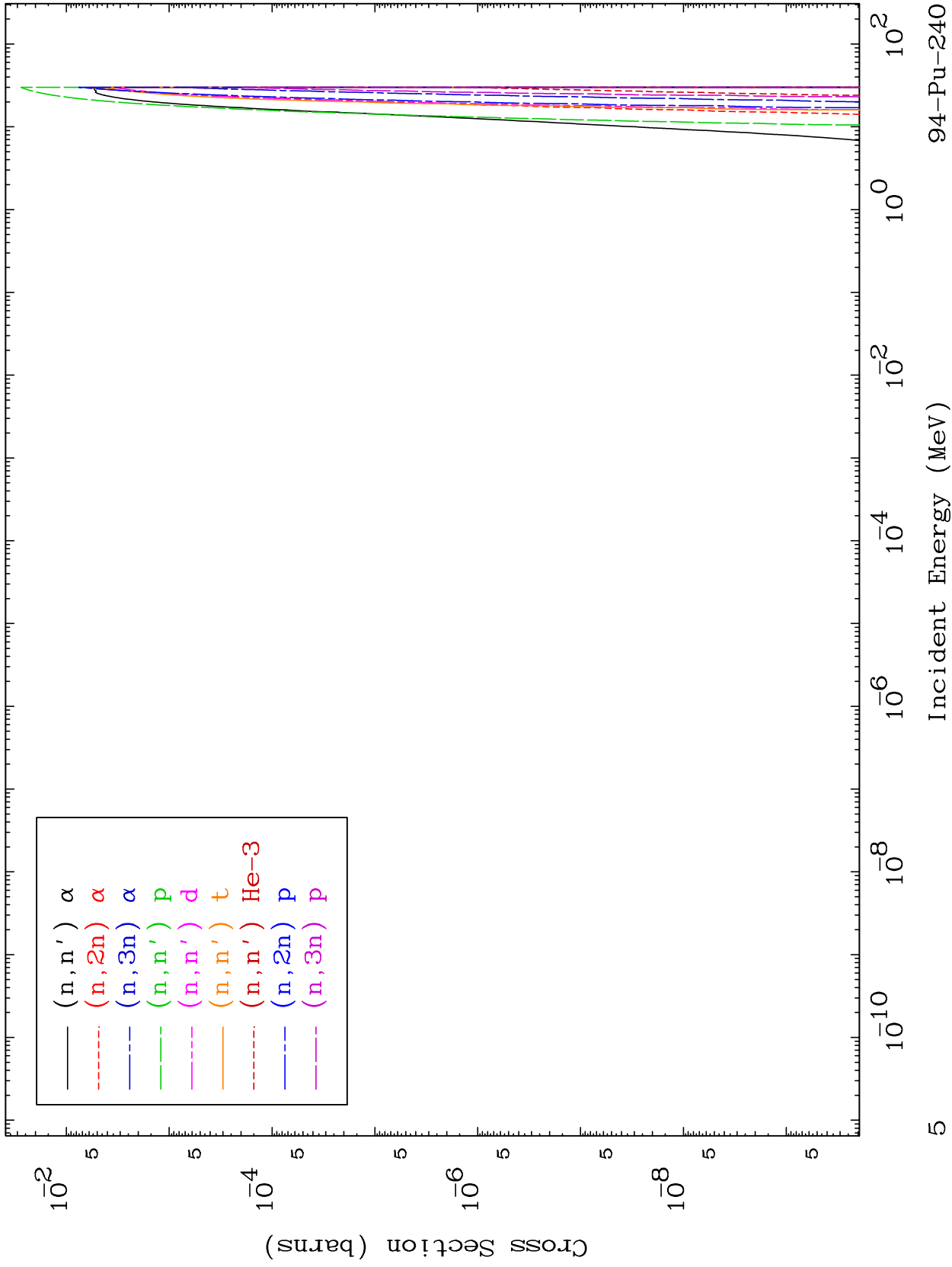
94-Pu-240



MAT 9440

Charged Particle  
293 Kelvin Cross Sections

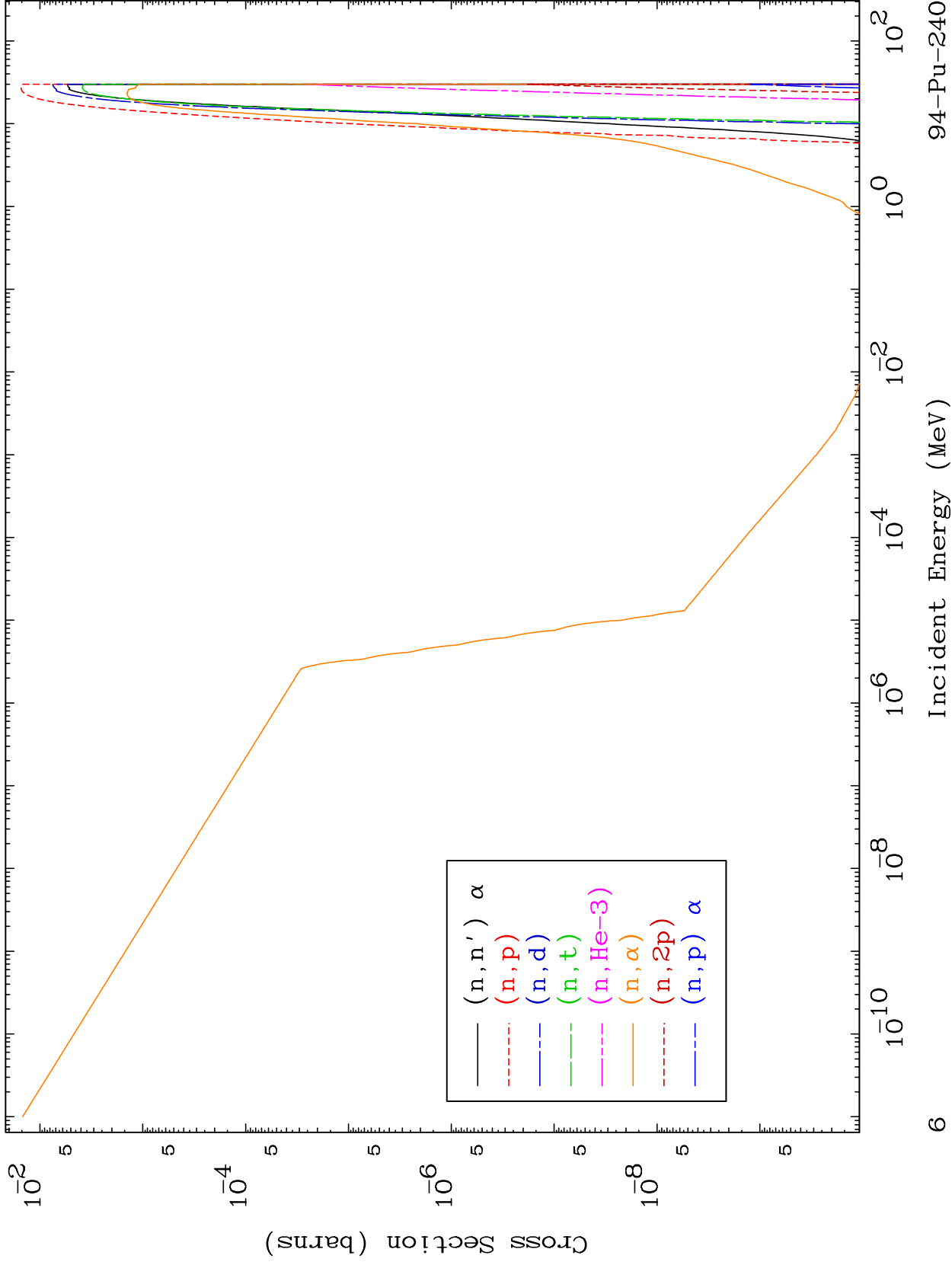
94-Pu-240



MAT 9440

Charged Particle  
293 Kelvin Cross Sections

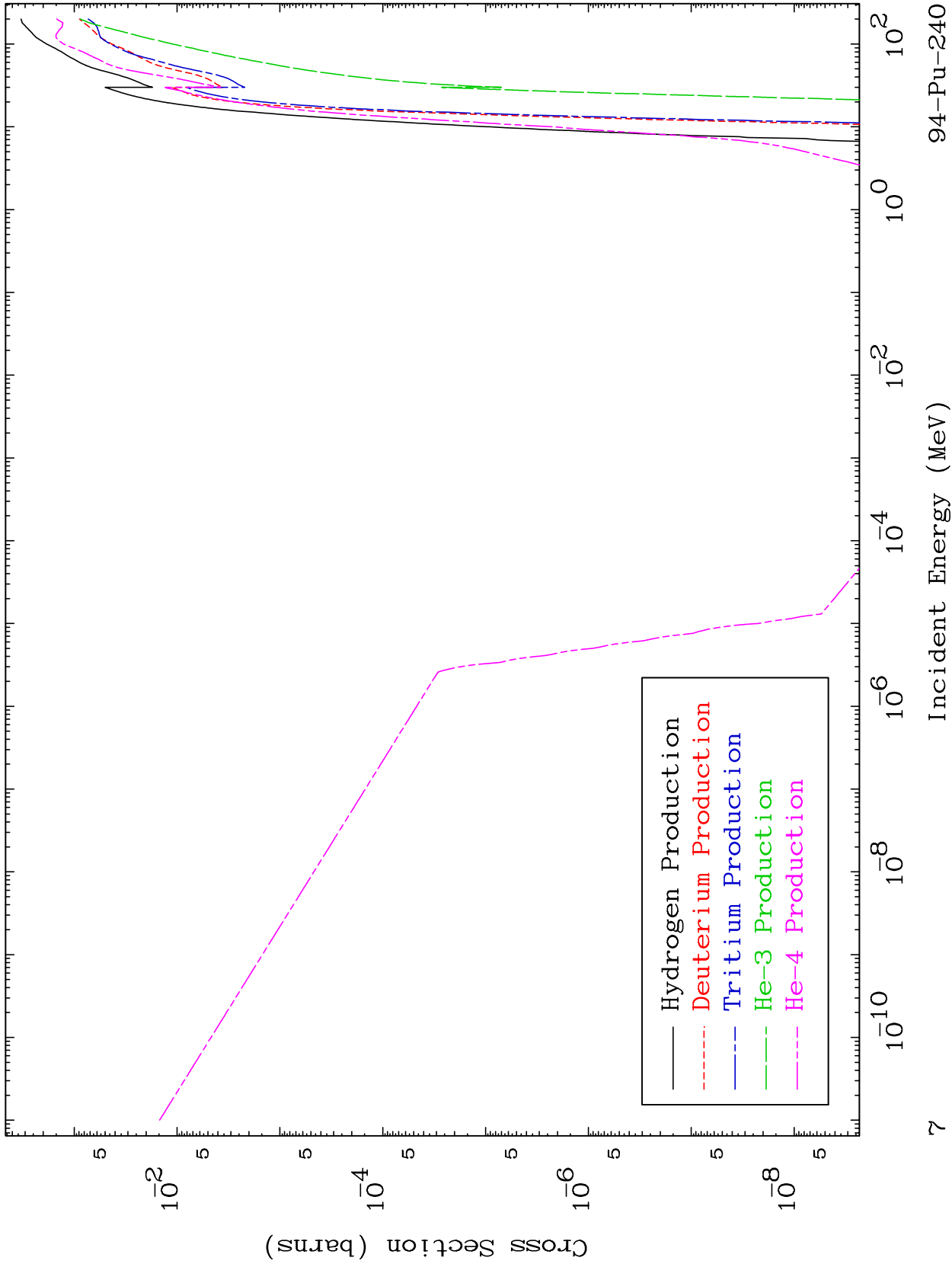
94-Pu-240



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Particle Production  
293 Kelvin Cross Sections

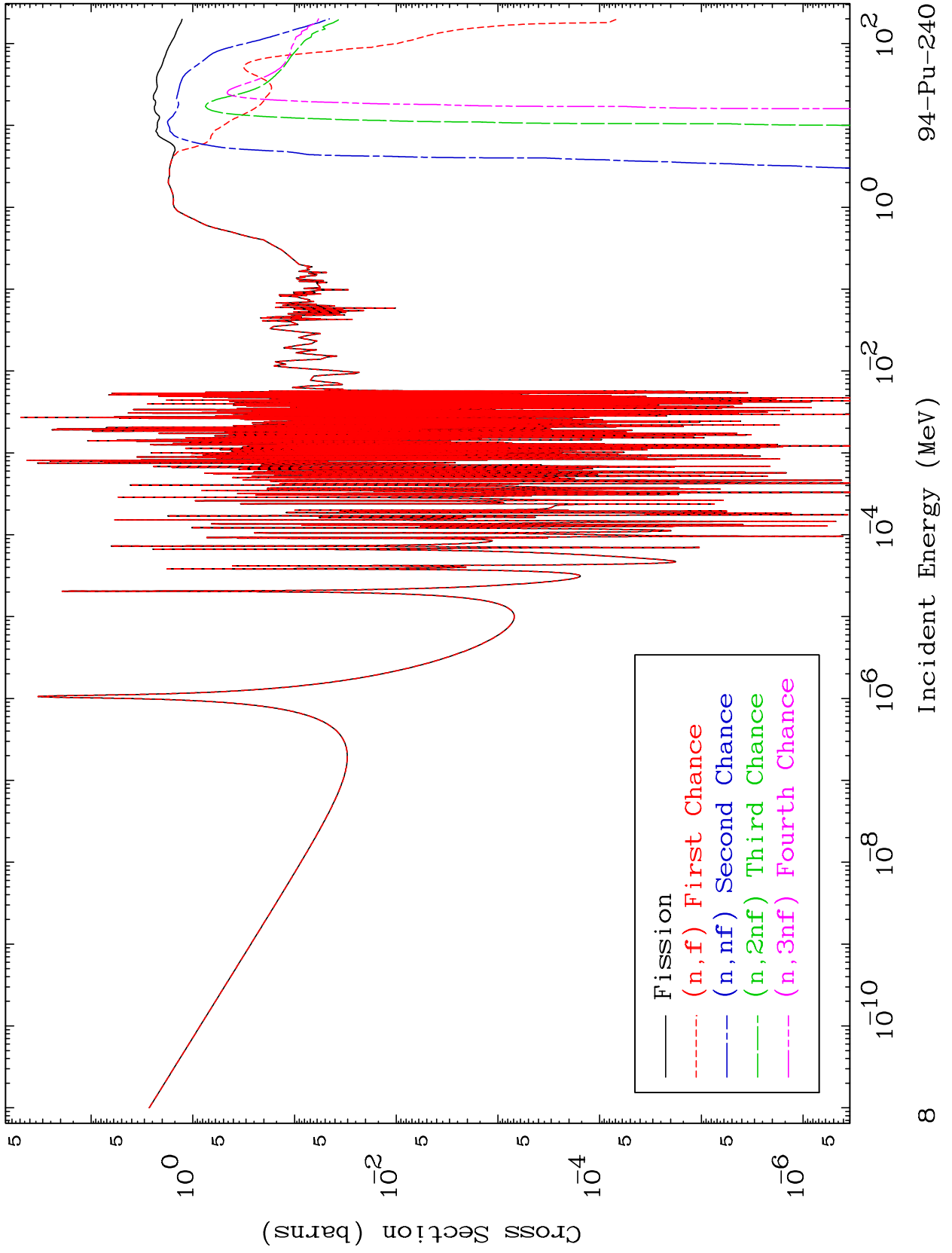
94-Pu-240



MAT 9440

Fission  
293 Kelvin Cross Sections

94-Pu-240

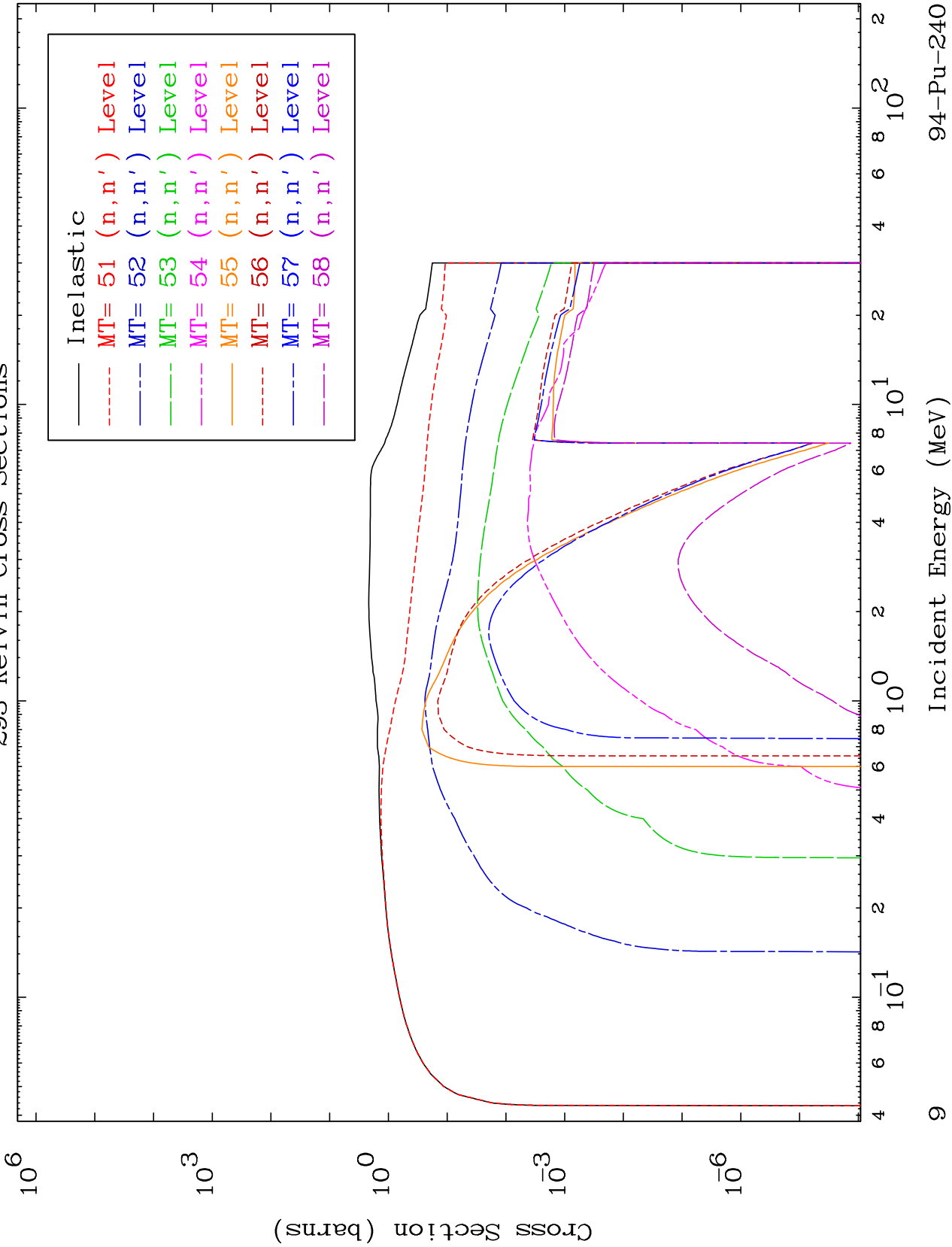


MAT 9440

(n,n') Levels

94-Pu-240

293 Kelvin Cross Sections

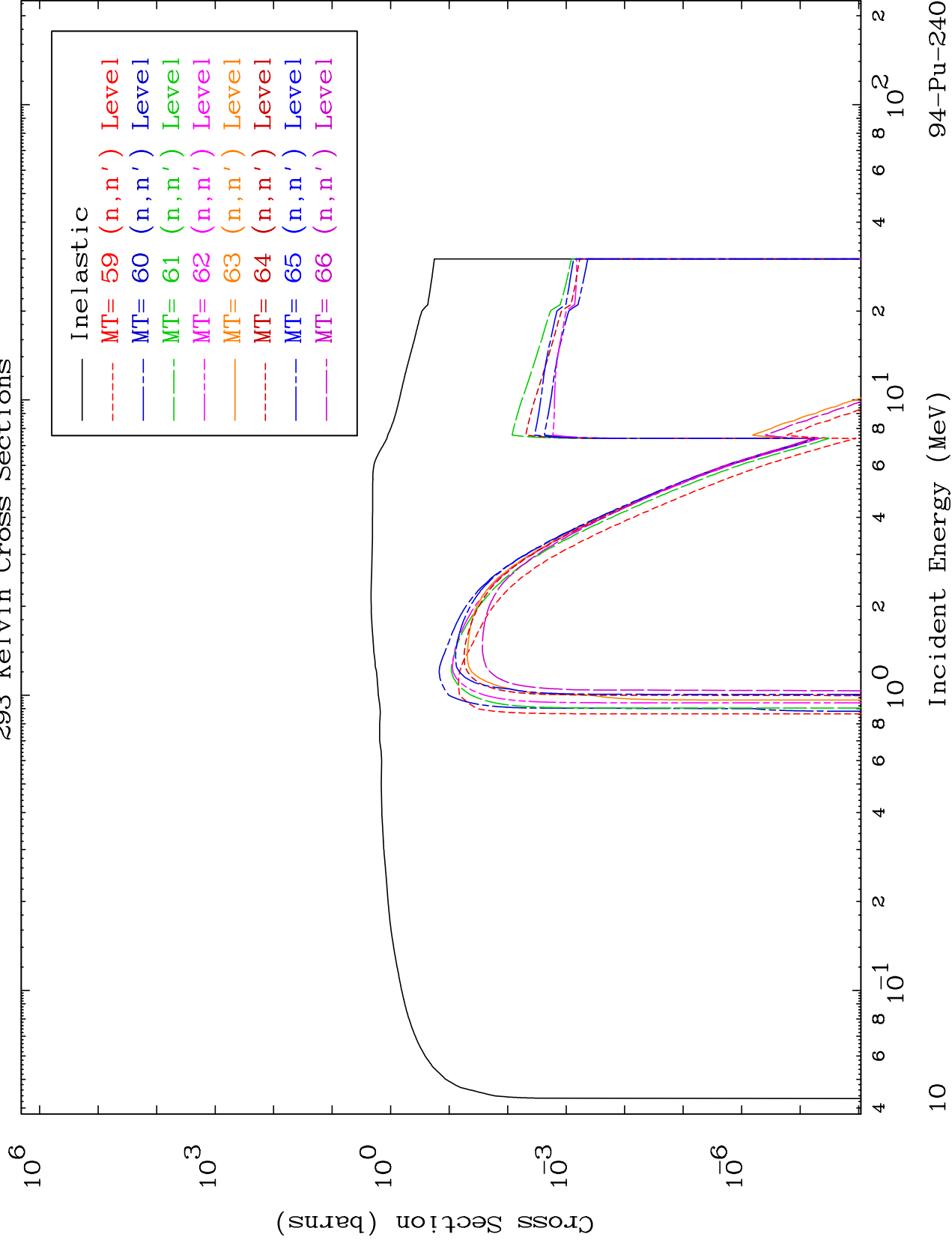


MAT 9440

(n,n') Levels

293 Kelvin Cross Sections

94-Pu-240



10

Incident Energy (MeV)

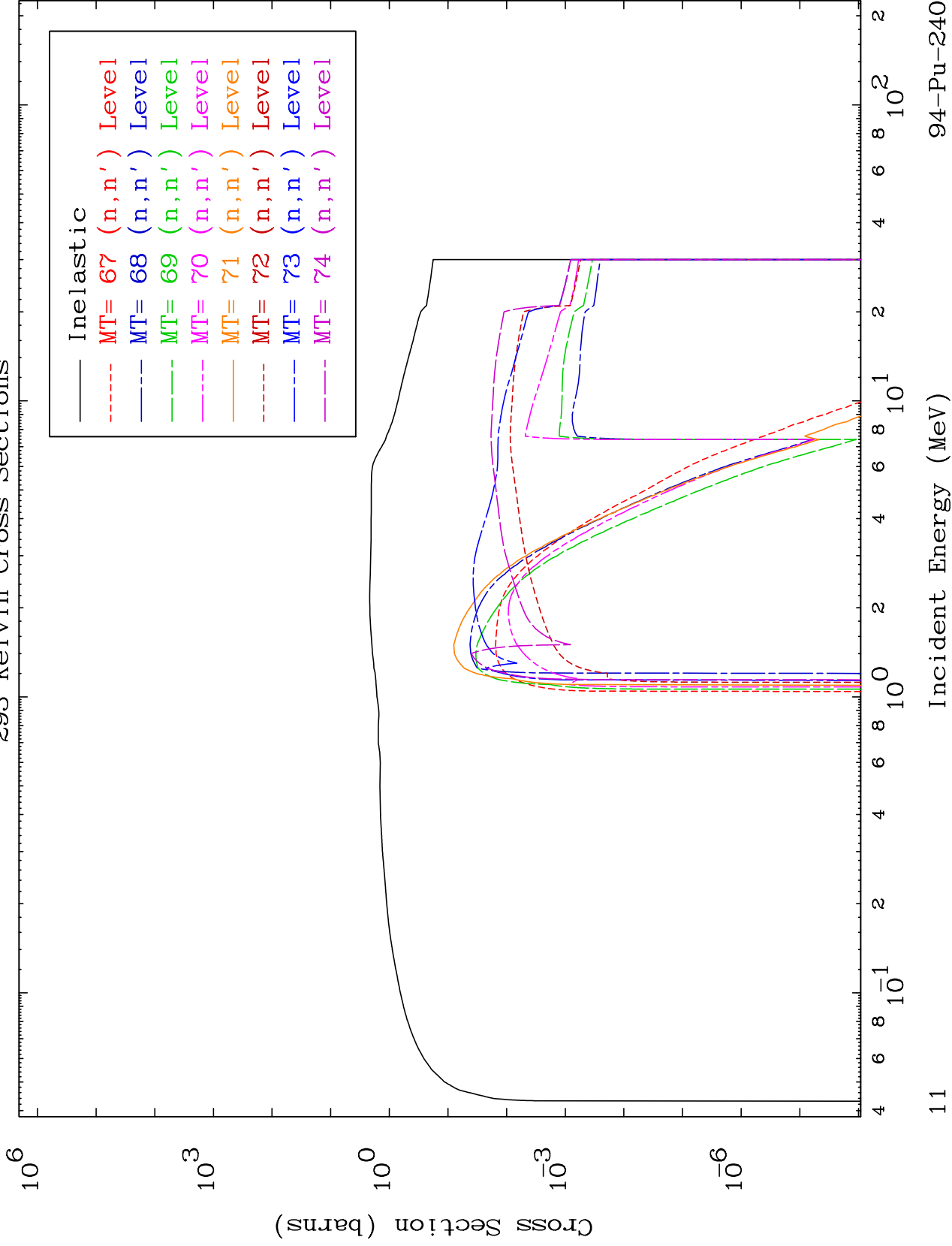
94-Pu-240

MAT 9440

(n,n') Levels

94-Pu-240

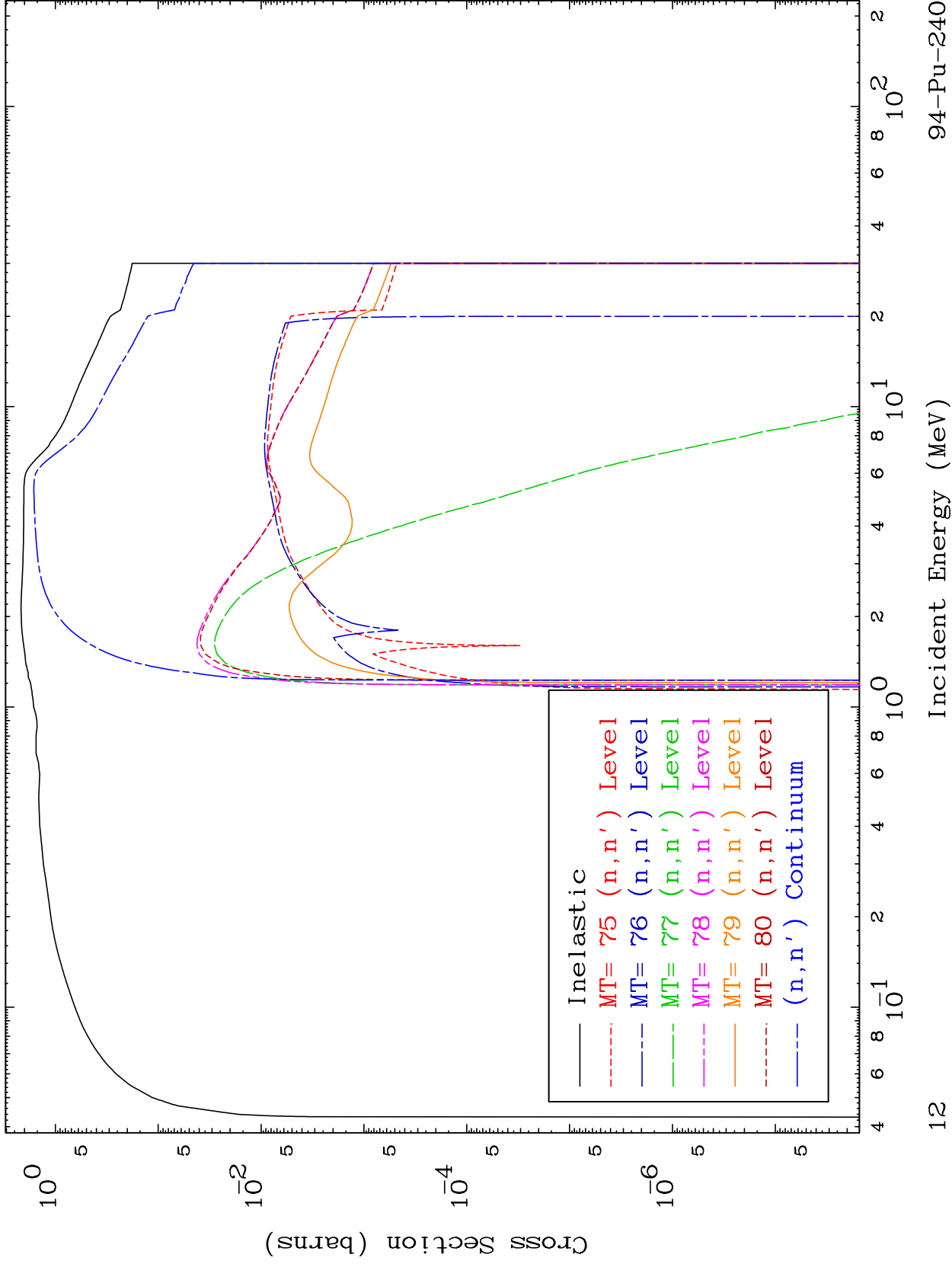
293 Kelvin Cross Sections



MAT 9440

293 Kelvin Cross Sections  
(n,n') Levels

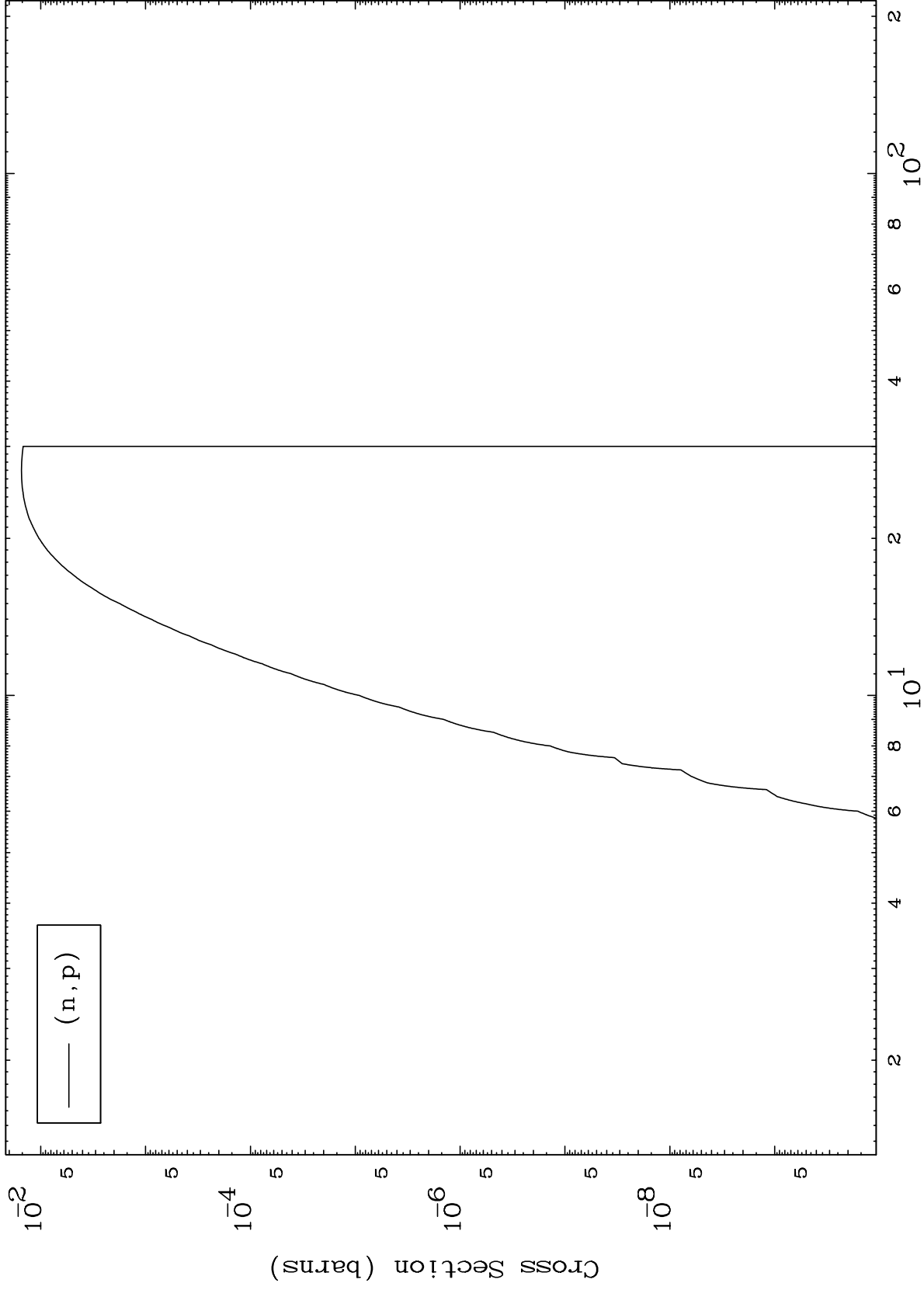
94-Pu-240



MAT 9440

(n,p) Levels  
293 Kelvin Cross Sections

94-Pu-240



13

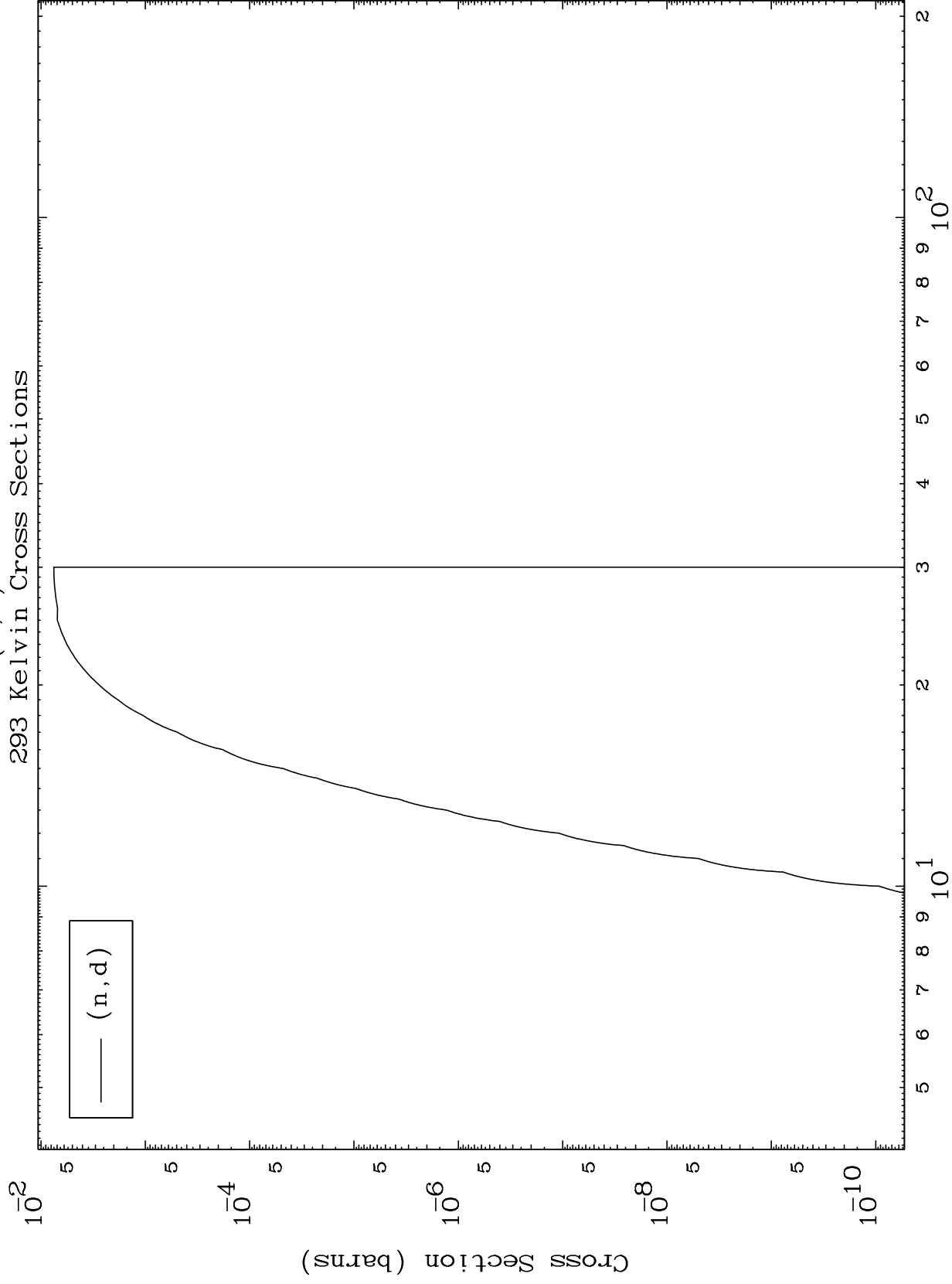
Incident Energy (MeV)

94-Pu-240

MAT 9440

(n,d) Levels  
293 Kelvin Cross Sections

94-Pu-240



14

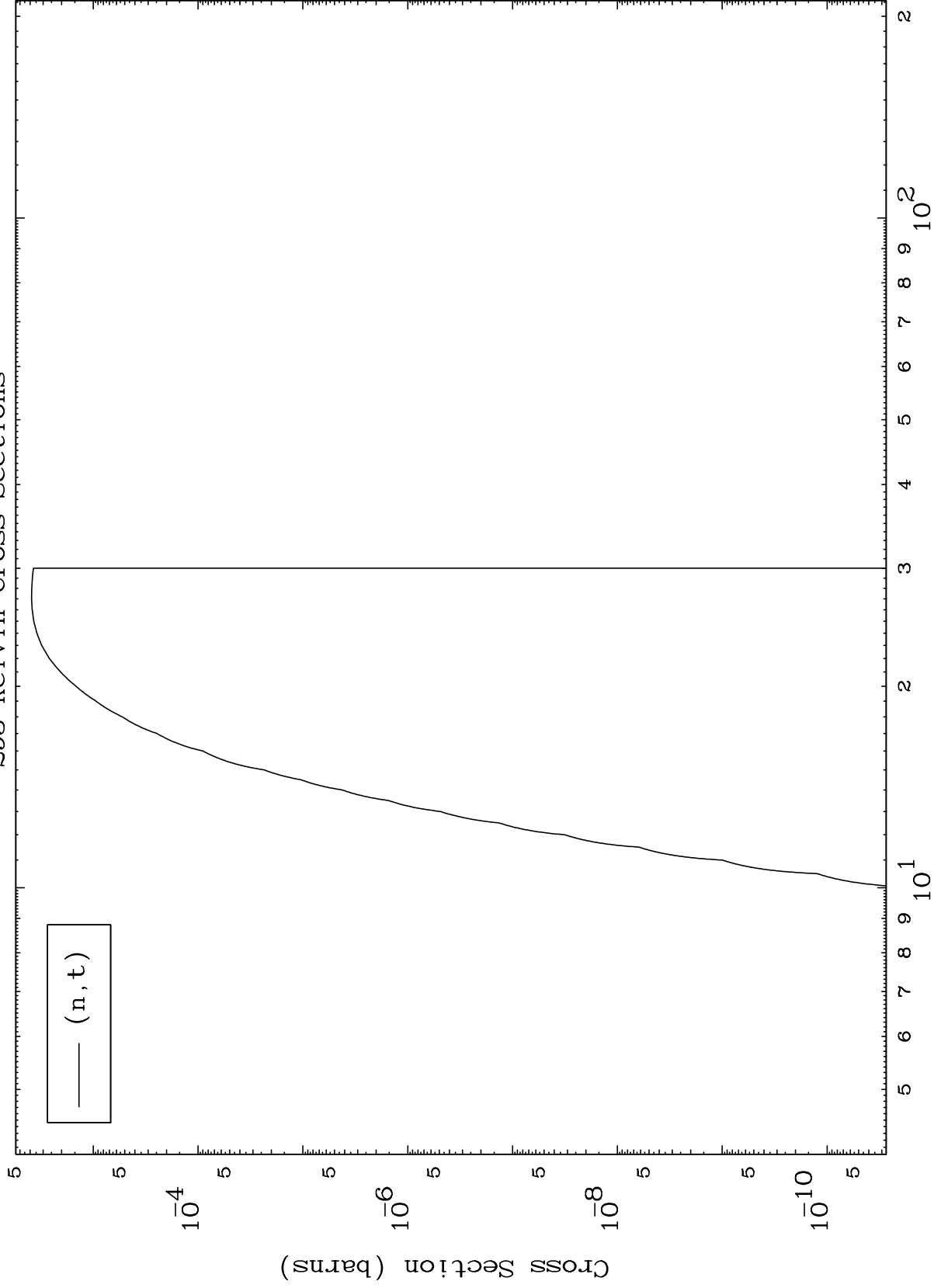
Incident Energy (MeV)

94-Pu-240

MAT 9440

(n,t) Levels  
293 Kelvin Cross Sections

94-Pu-240



15

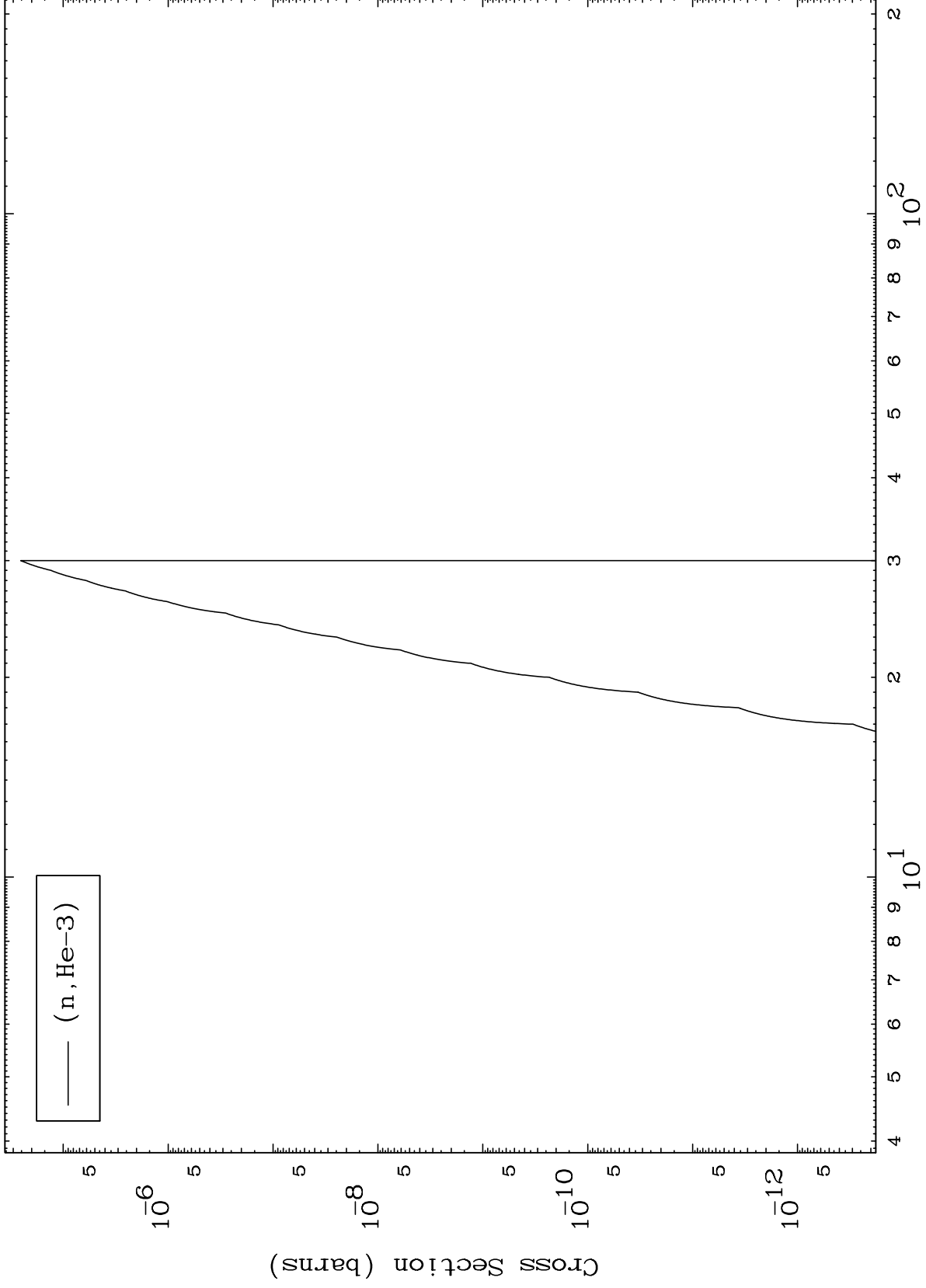
Incident Energy (MeV)

94-Pu-240

MAT 9440

(n,He3) Levels  
293 Kelvin Cross Sections

94-Pu-240



16

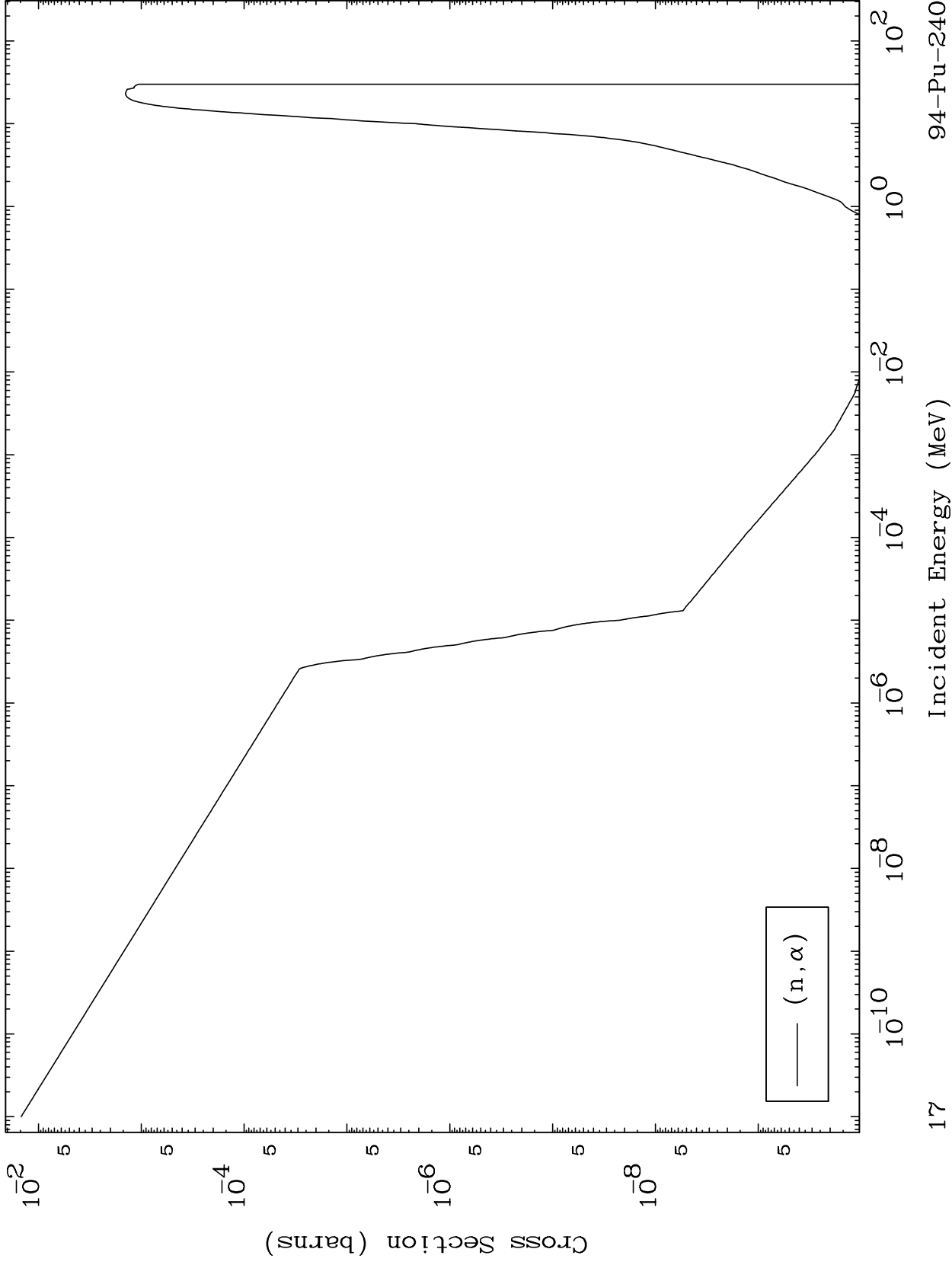
Incident Energy (MeV)

94-Pu-240

MAT 9440

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

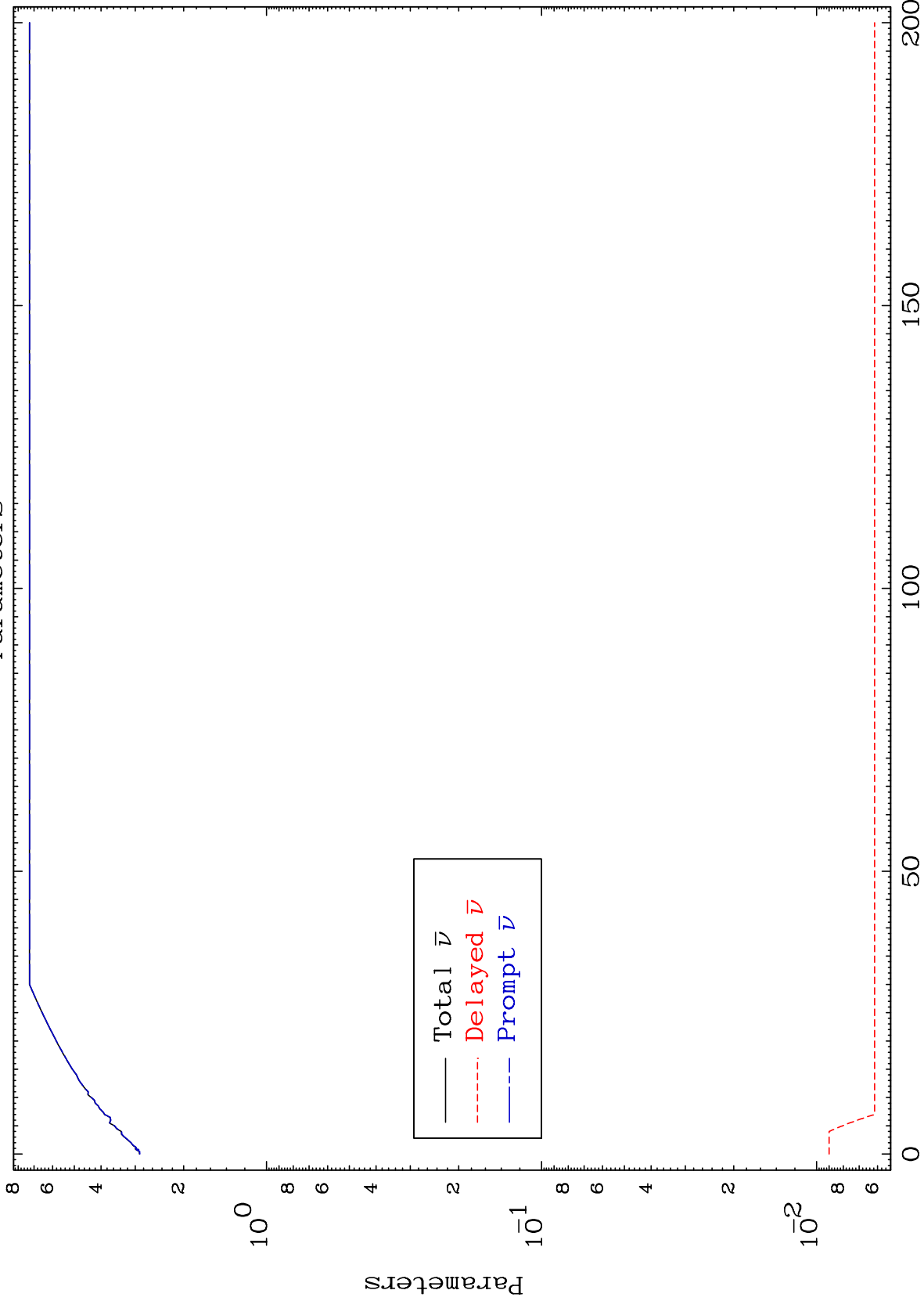
94-Pu-240



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Energy Release  
Parameters

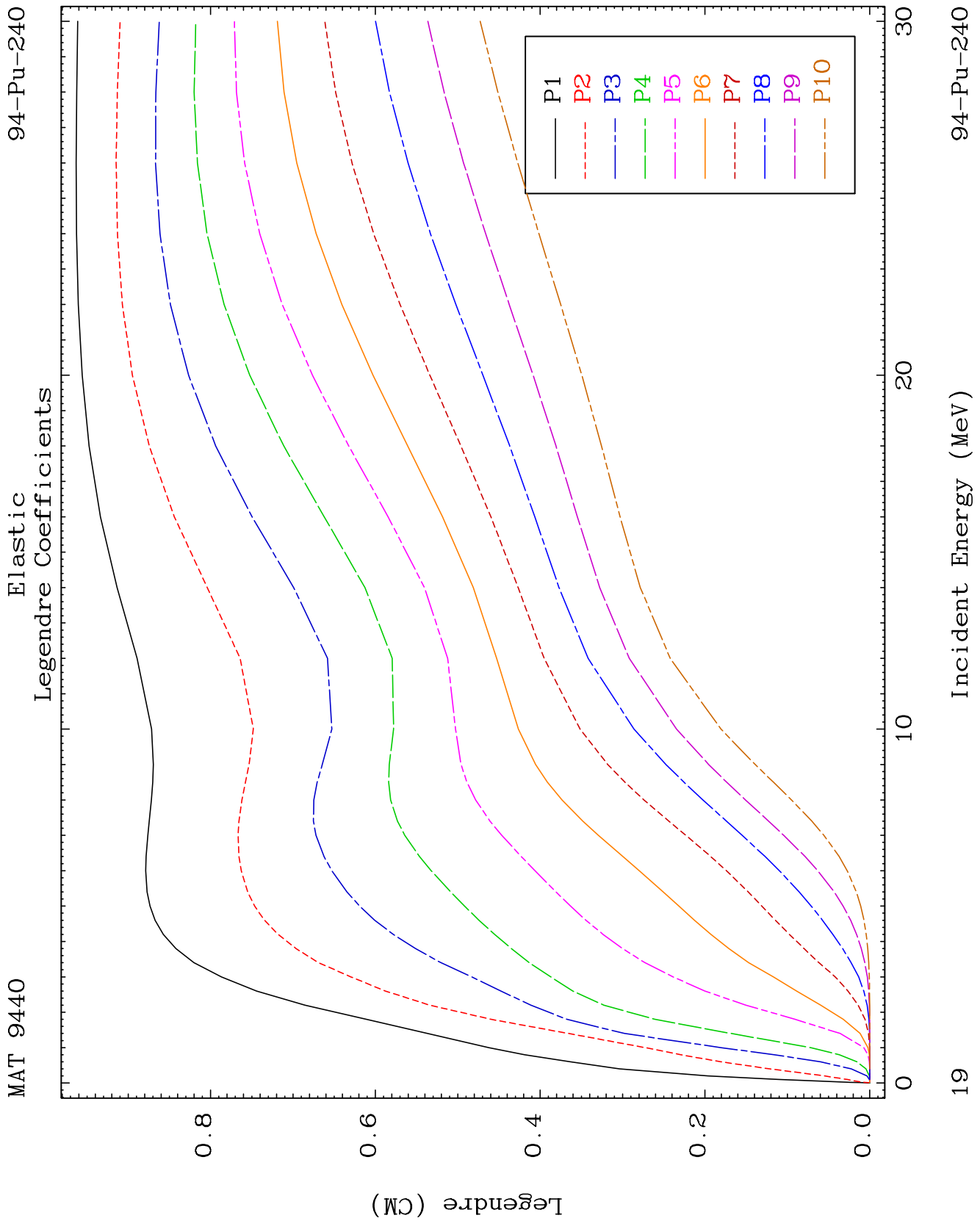
94-Pu-240

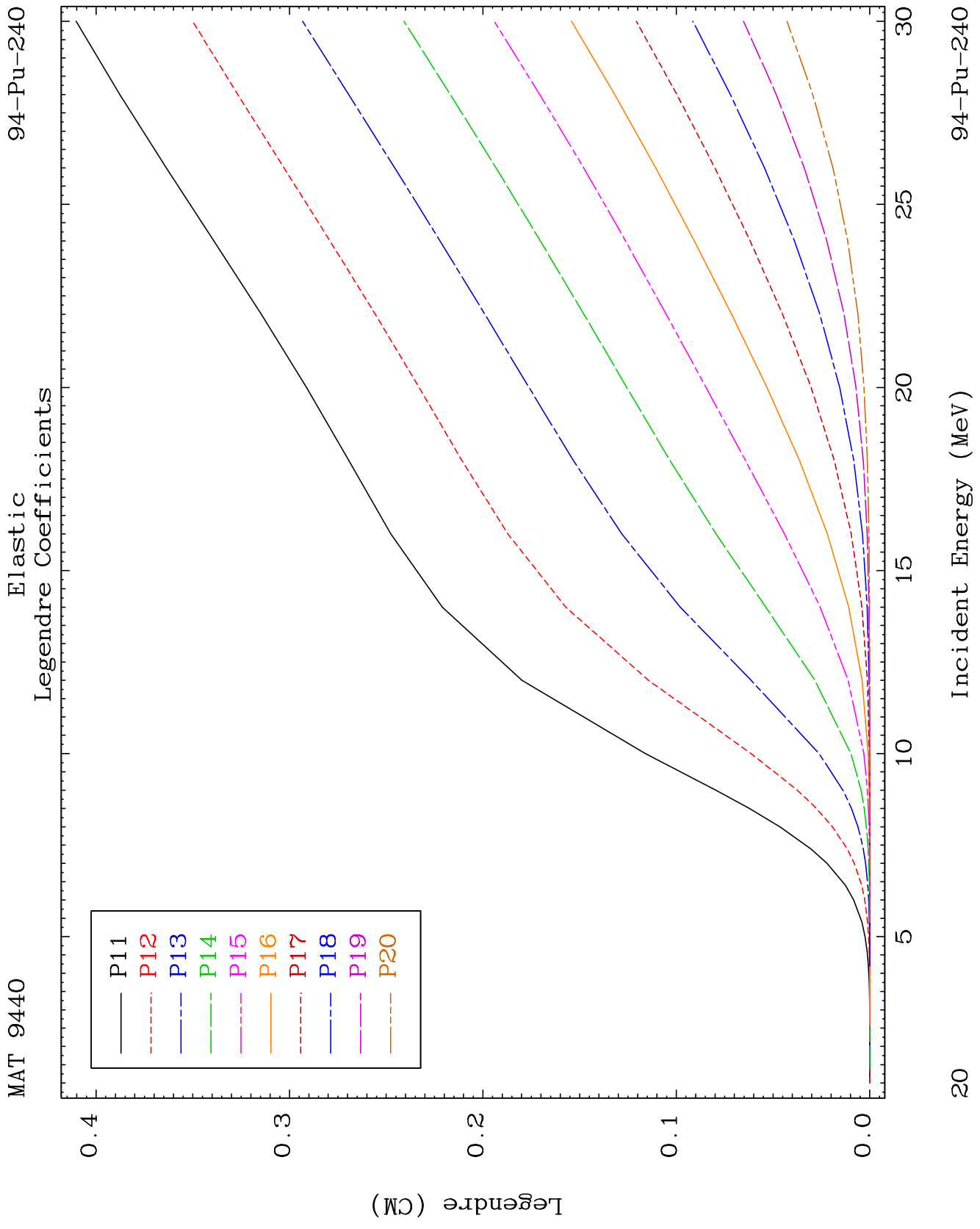


18

Incident Energy (MeV)

94-Pu-240

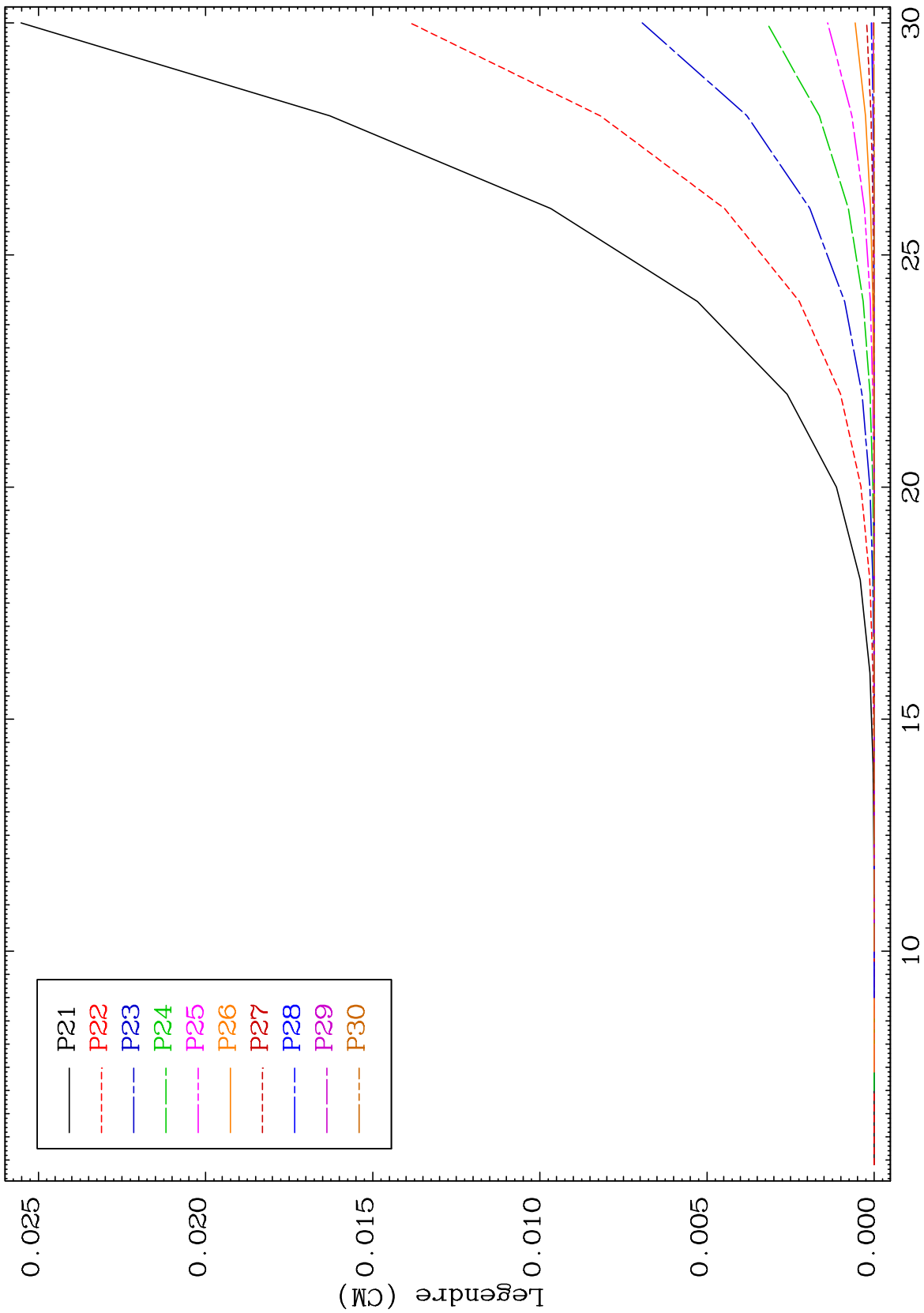




MAT 9440

Elastic  
Legendre Coefficients

94-Pu-240



21

Incident Energy (MeV)

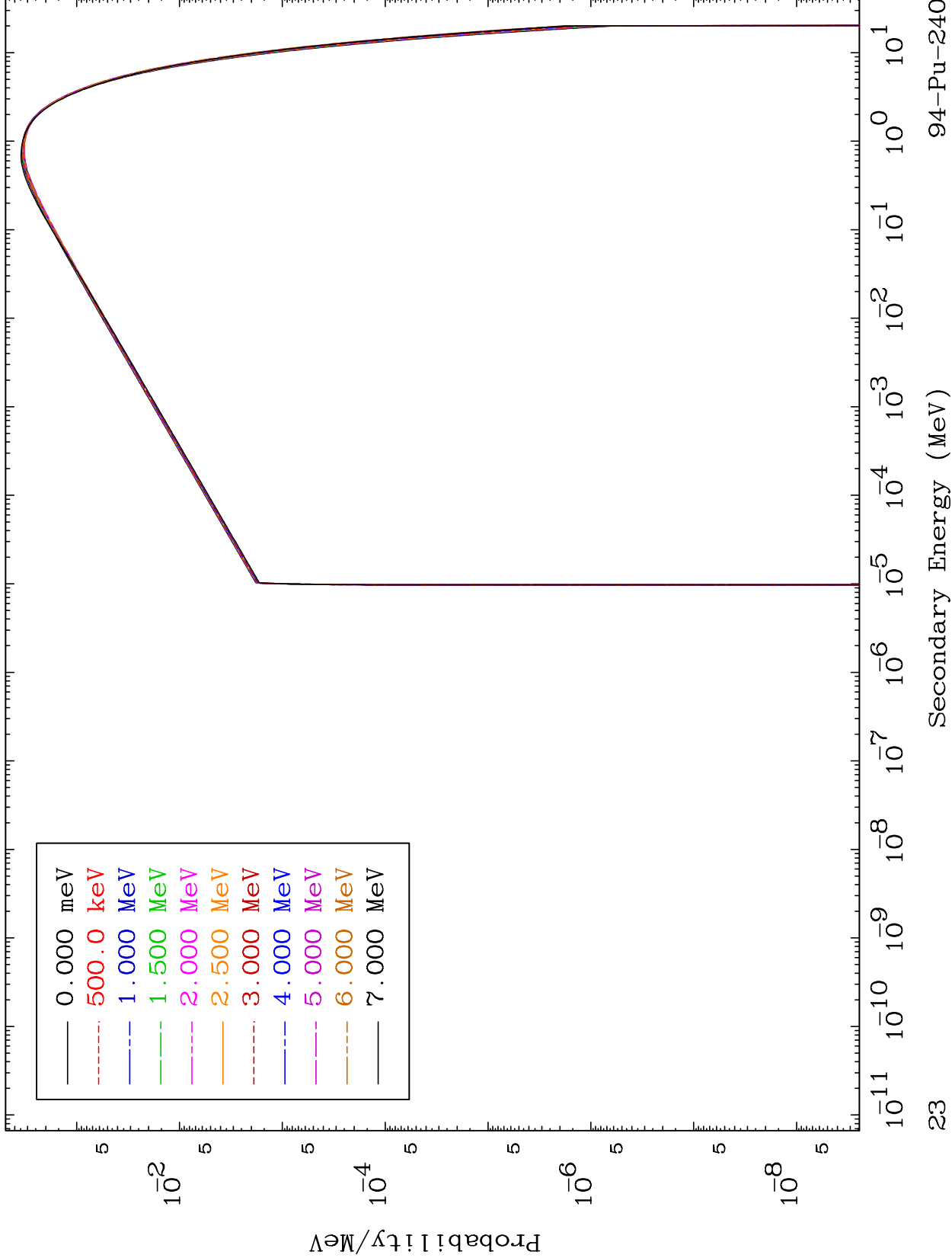
94-Pu-240



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### Fission Energy Distributions

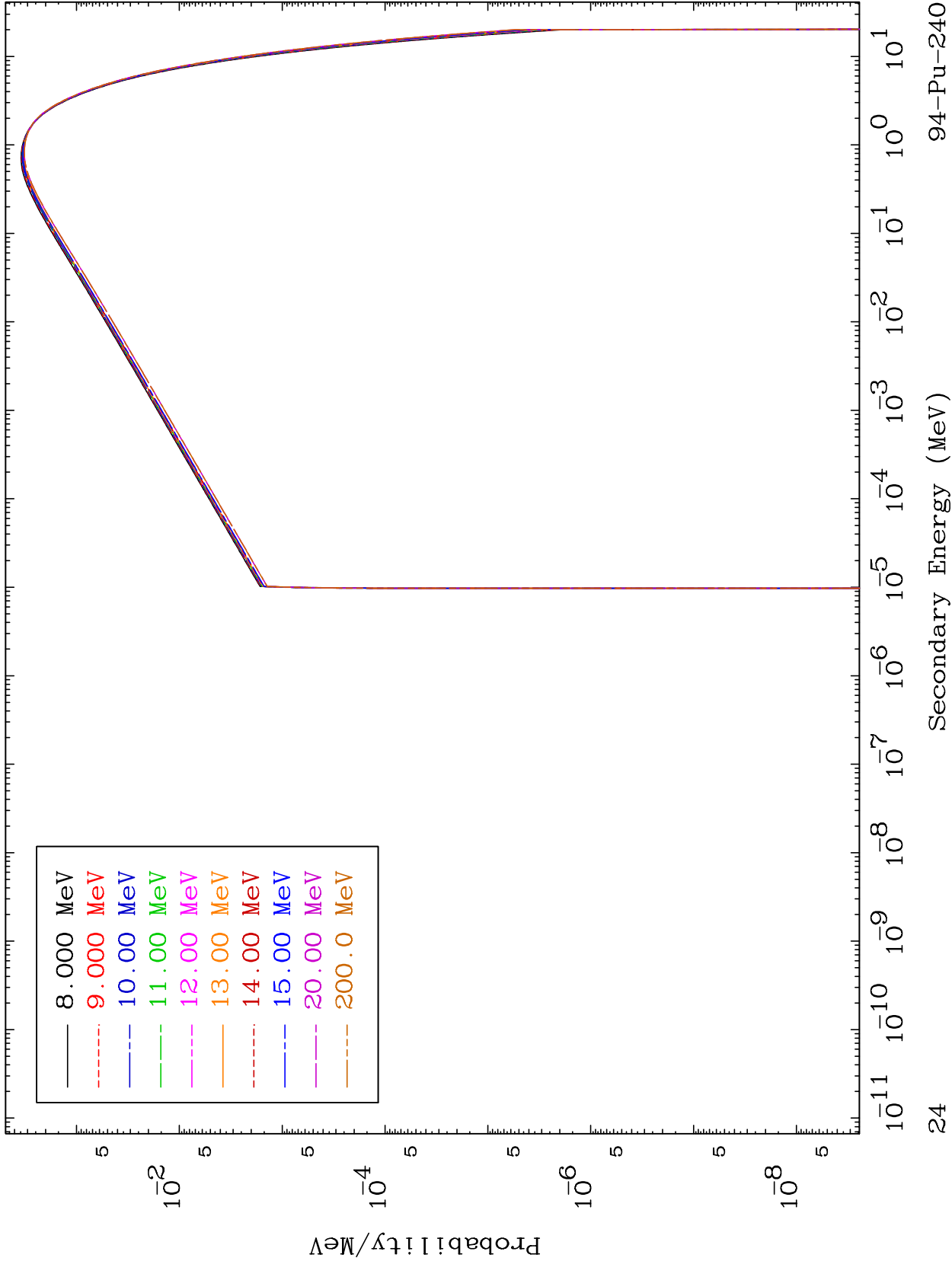
94-Pu-240



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### Fission Energy Distributions

94-Pu-240



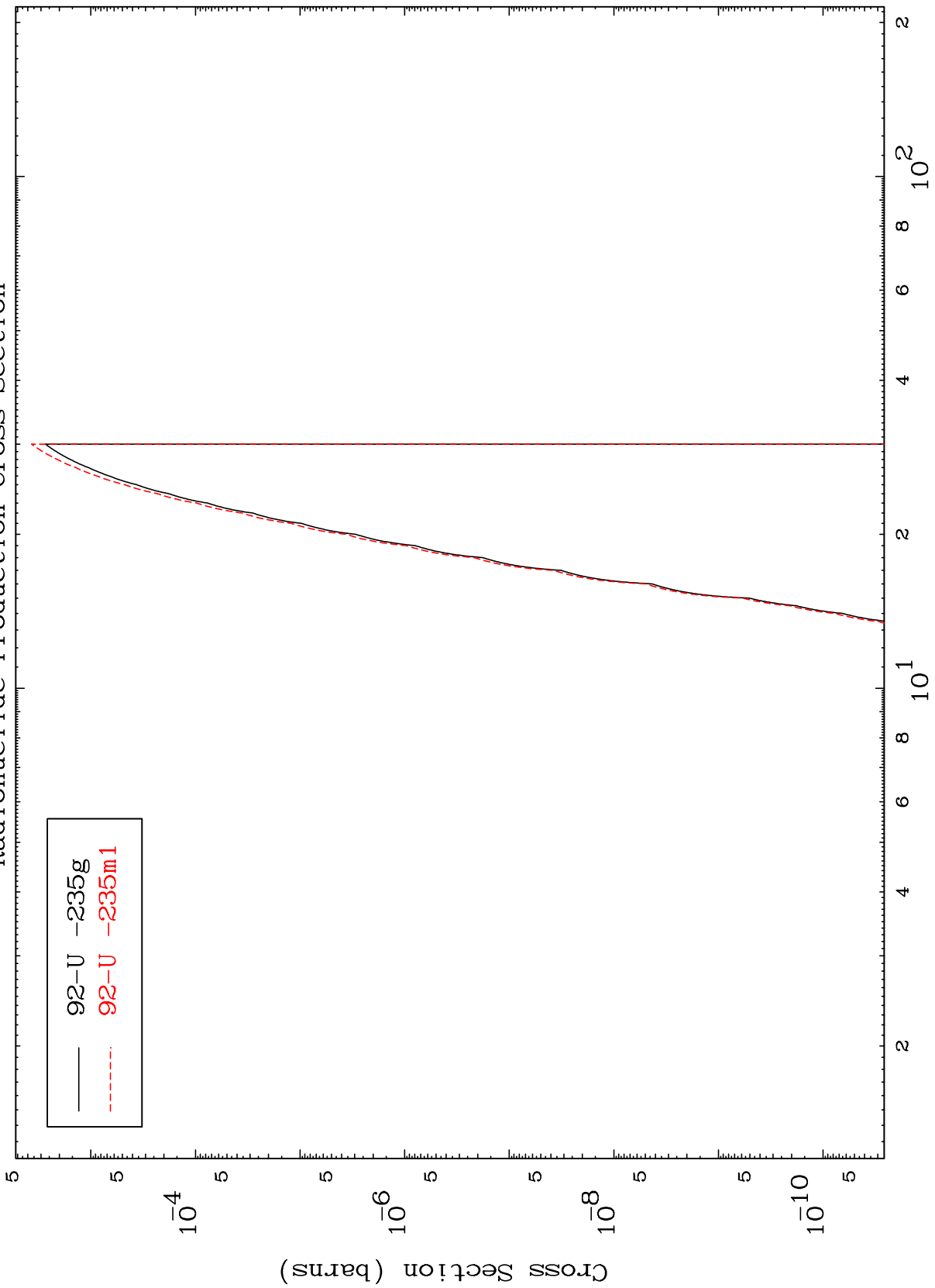
24

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(n,2n)  $\alpha$

94-Pu-240

Radionuclide Production Cross Section



92-U -235g  
92-U -235m1

Incident Energy (MeV)

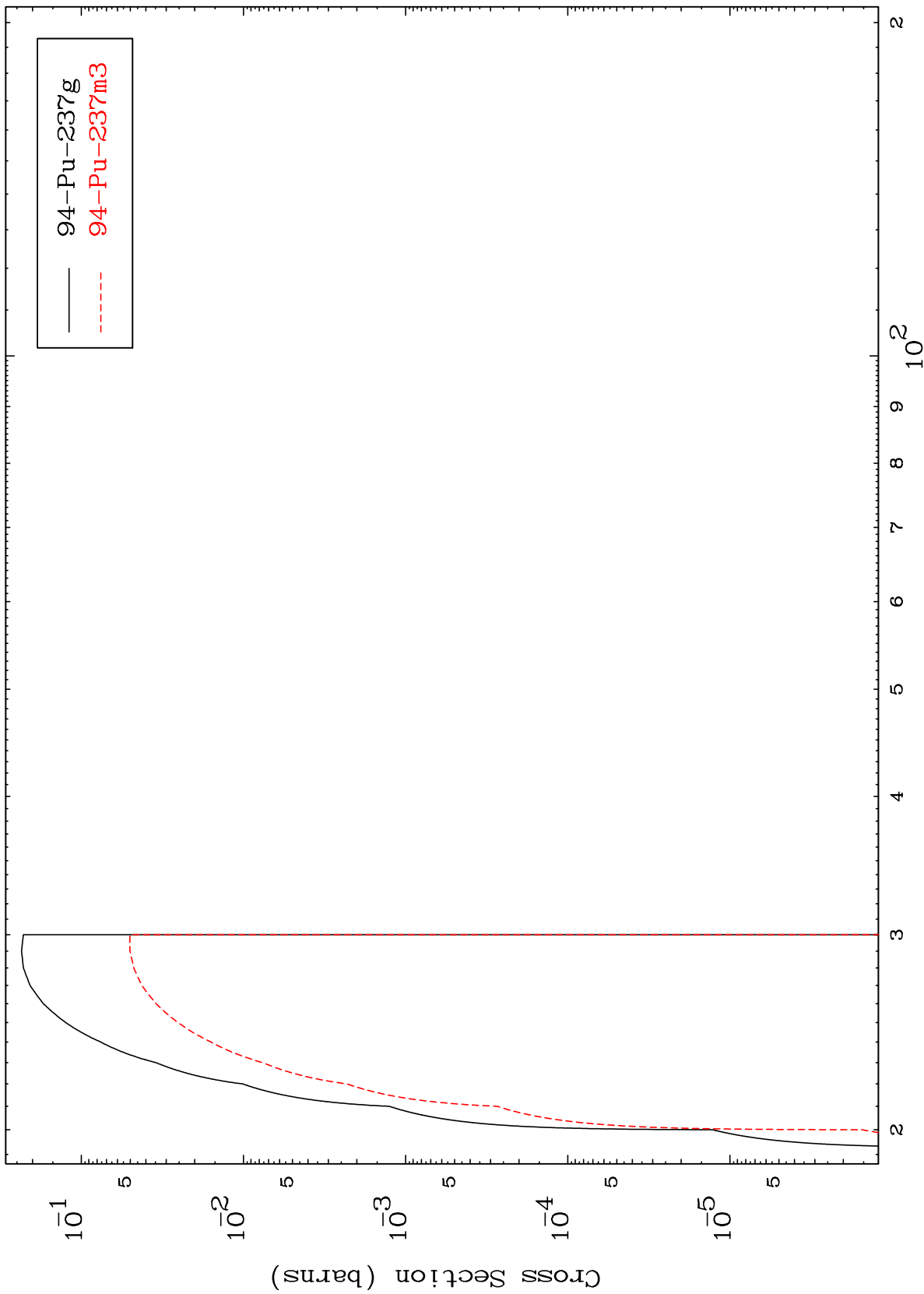
94-Pu-240

25

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94-Pu-240

(n,4n)  
Radionuclide Production Cross Section



94-Pu-240

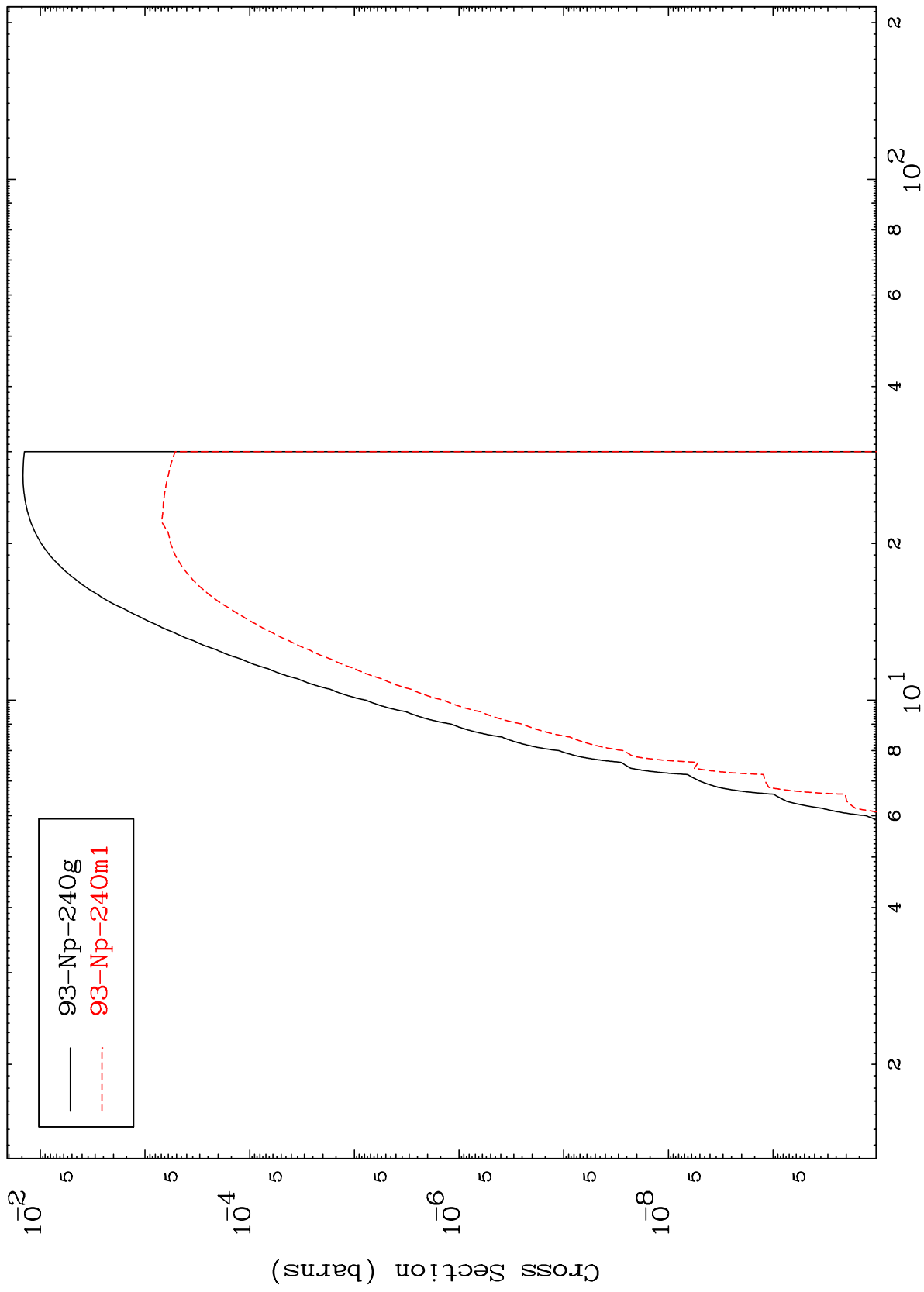
Incident Energy (MeV)

26

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94-Pu-240

(n,p)  
Radionuclide Production Cross Section



27

Incident Energy (MeV)

94-Pu-240