

Program EVALPLOT
(Version 2021-1)

by

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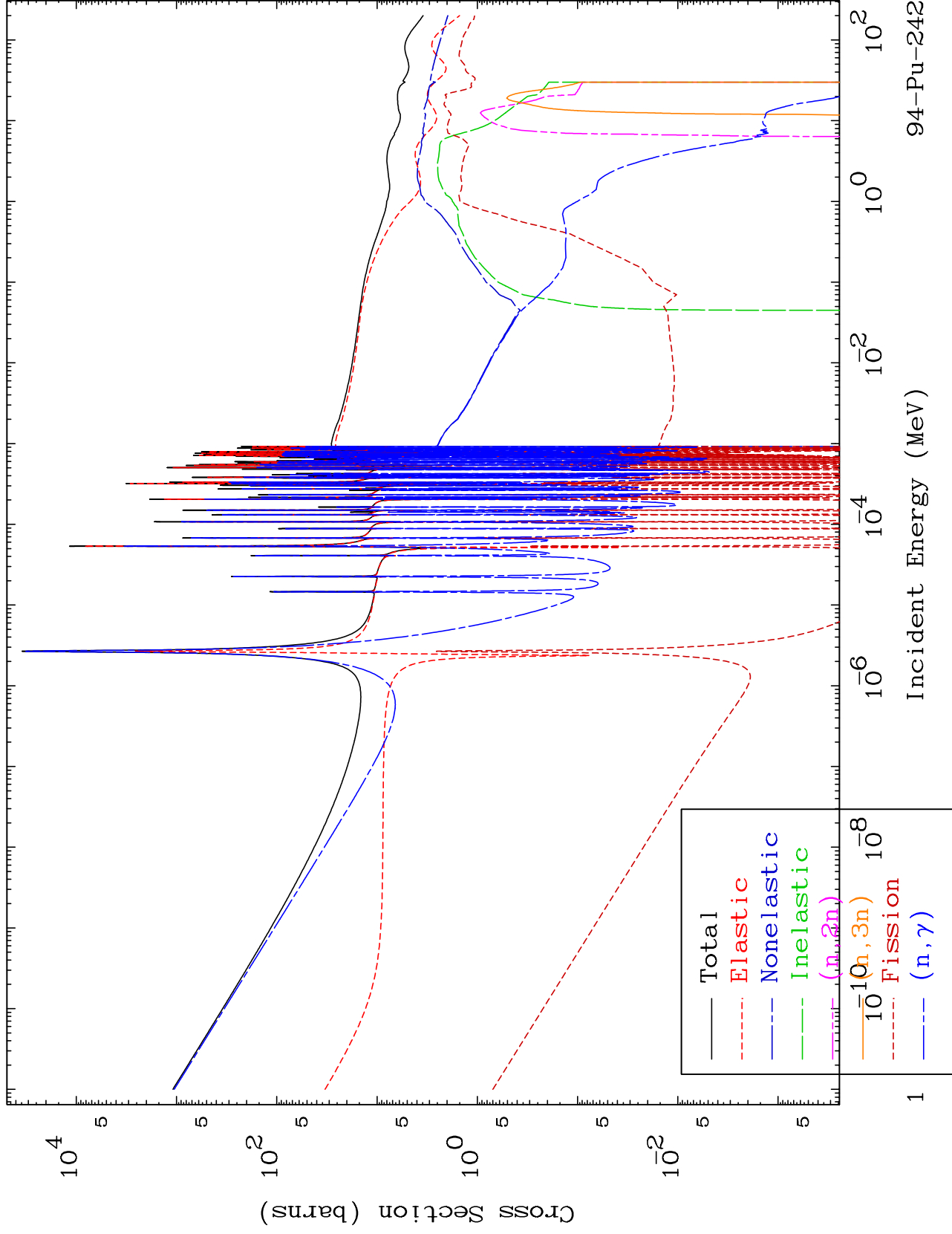
Press Mouse Button to Start

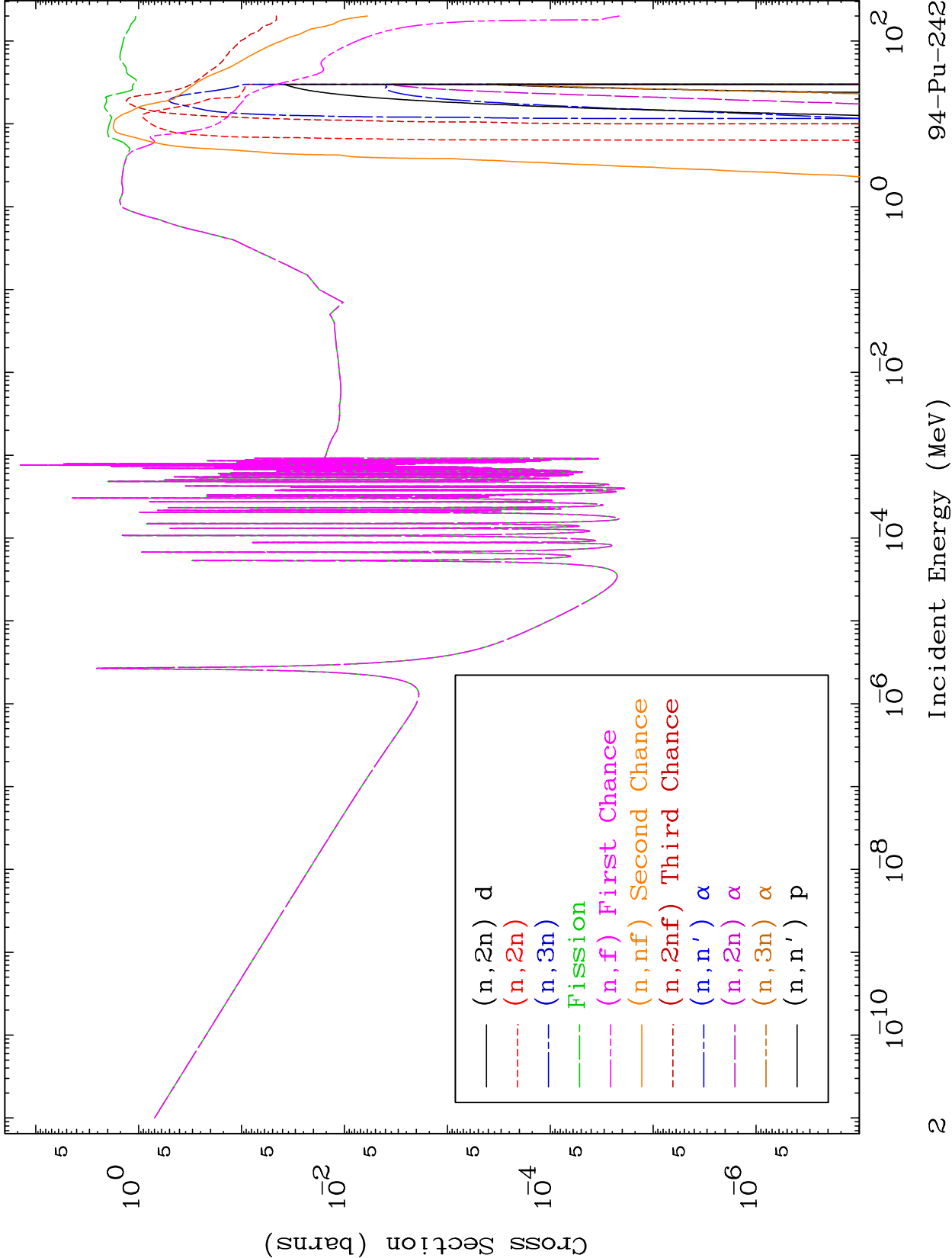
MAT 9446

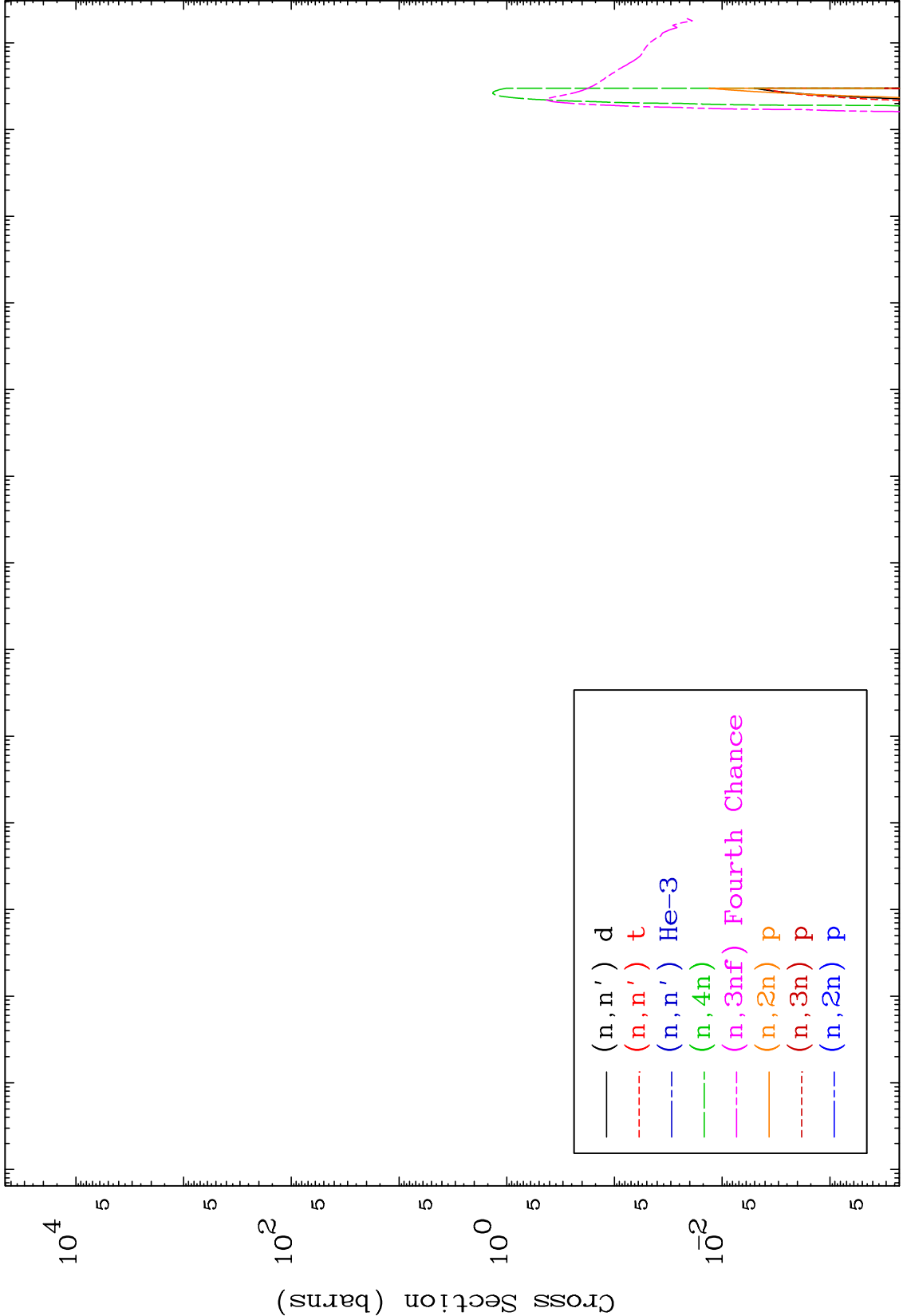
Neutron Major

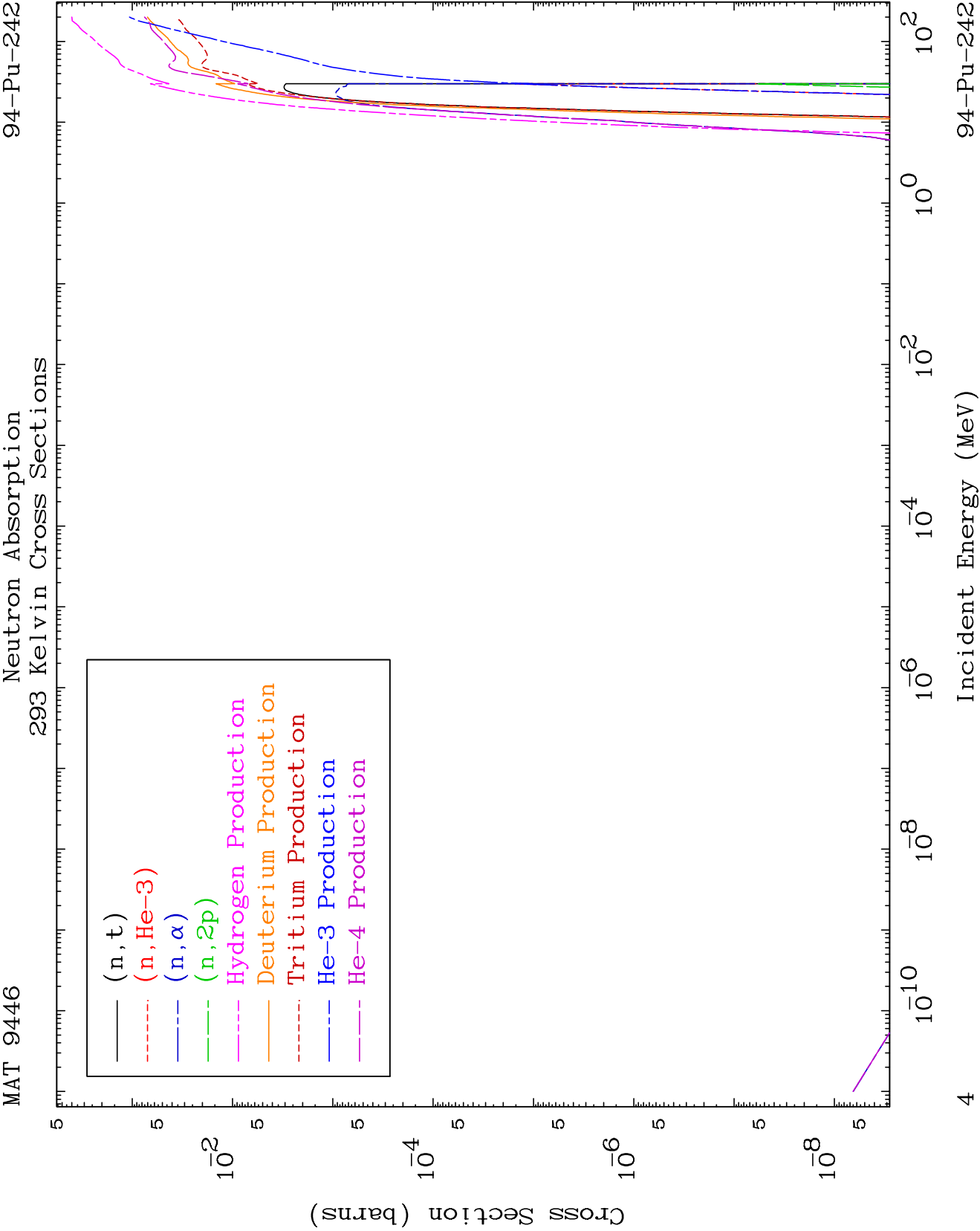
94-Pu-242

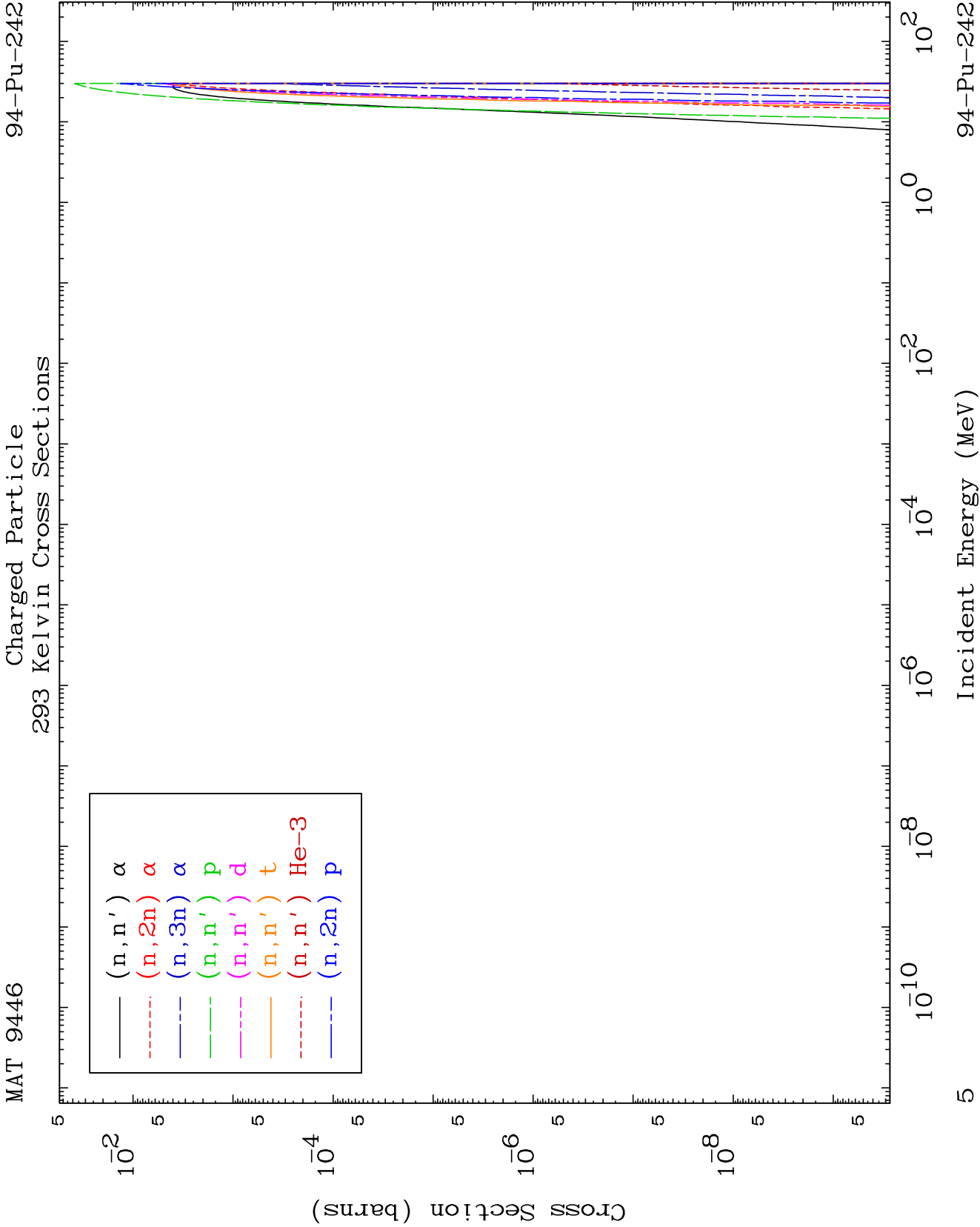
293 Kelvin Cross Sections

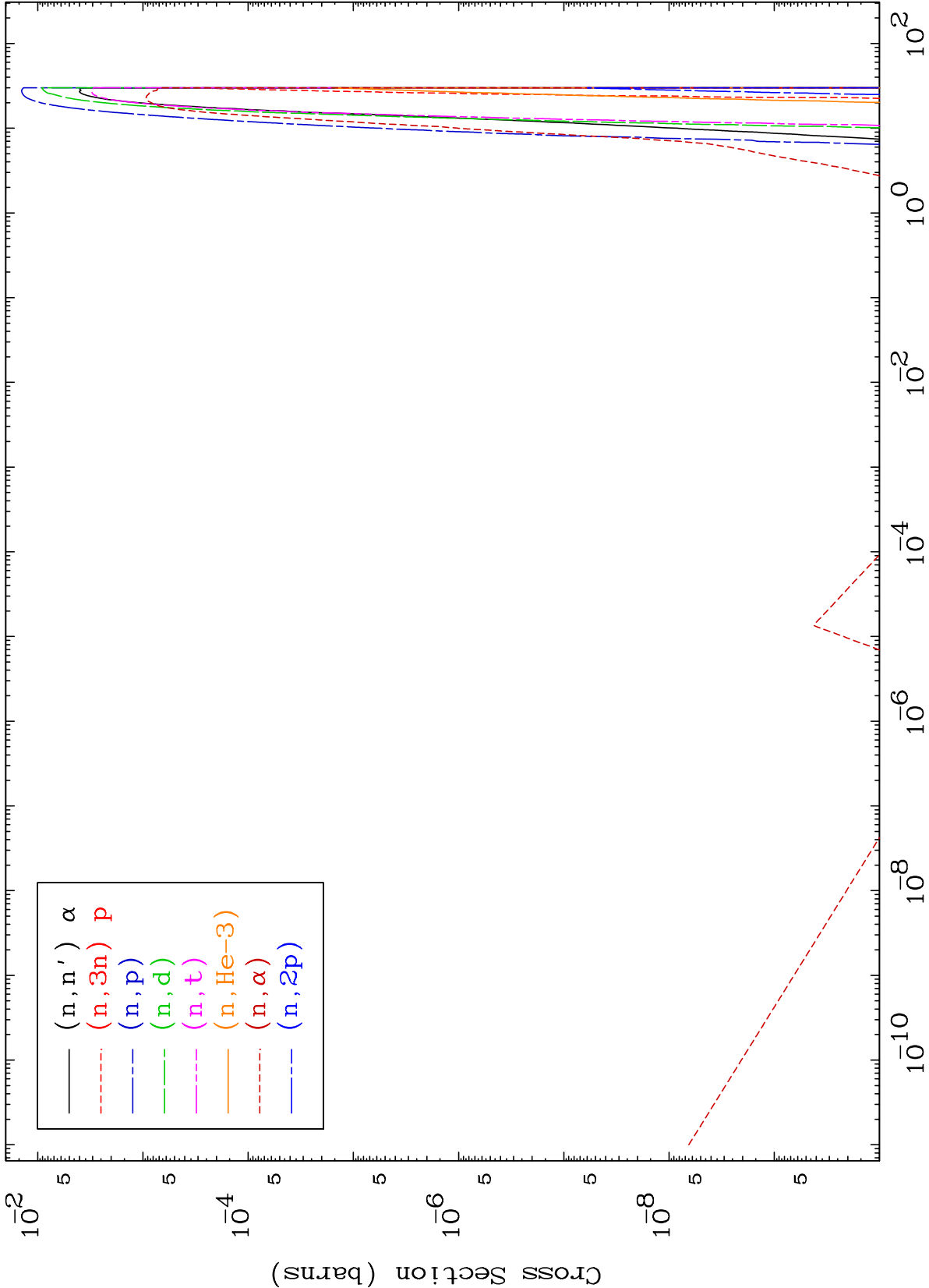


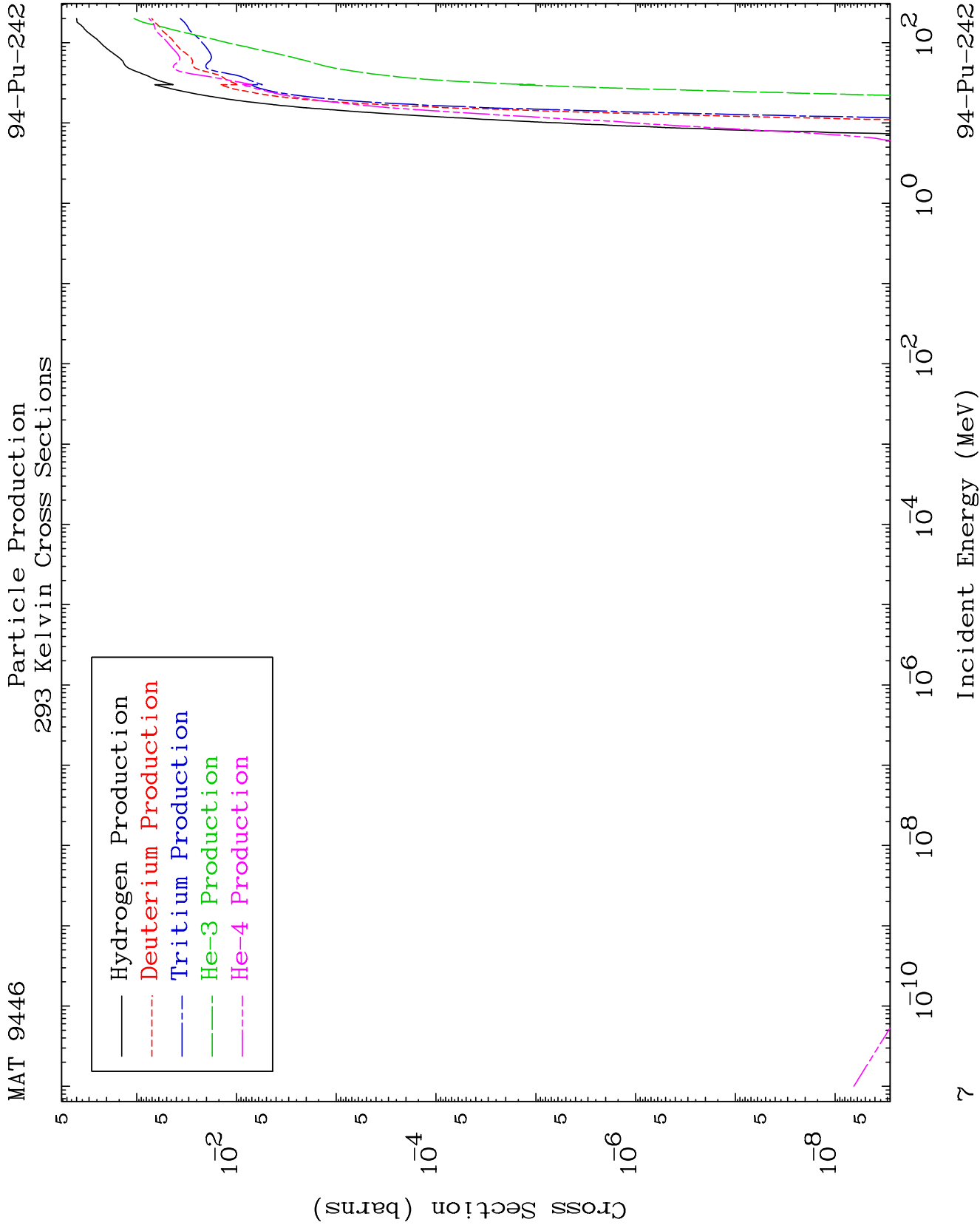










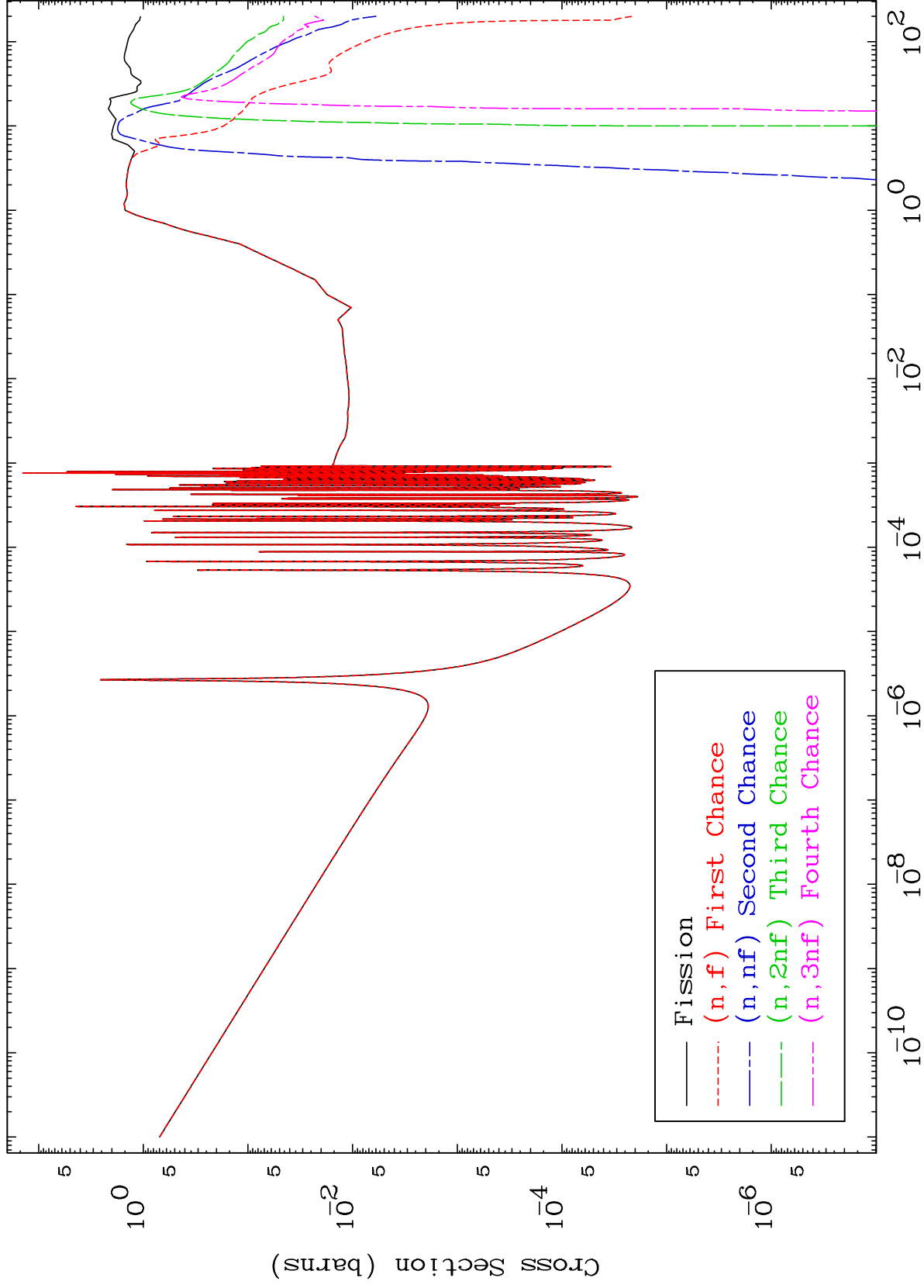


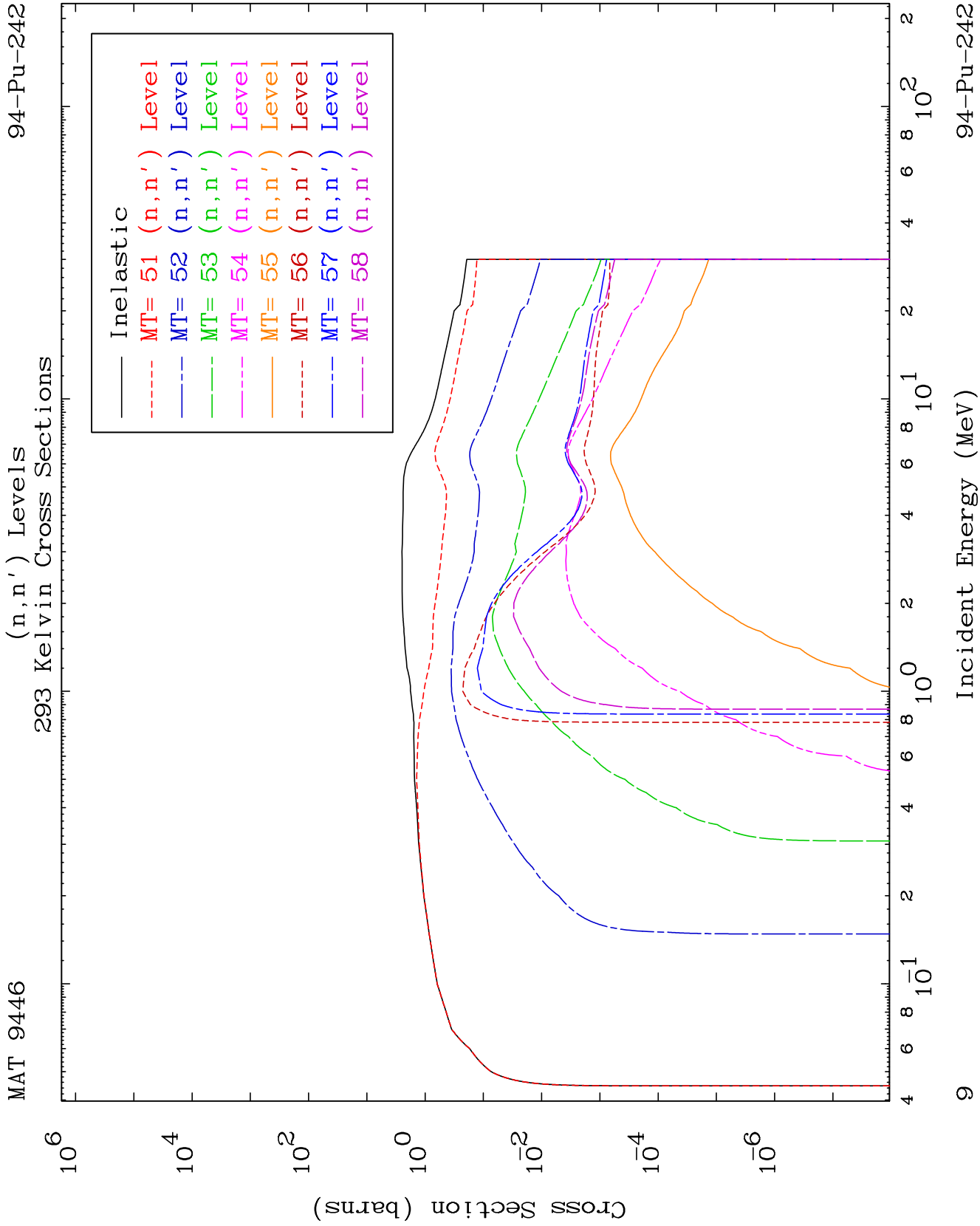
MAT 9446

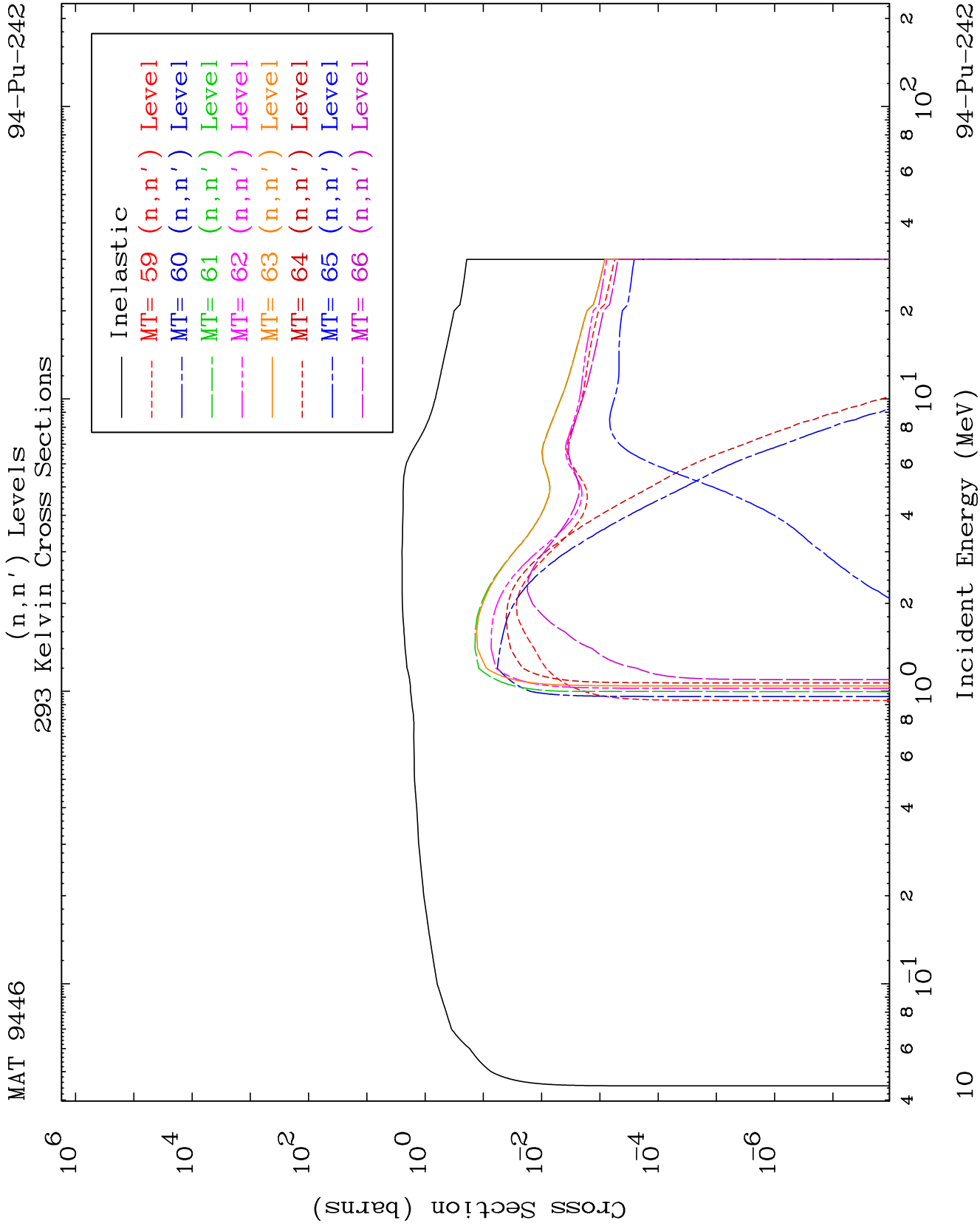
Fission

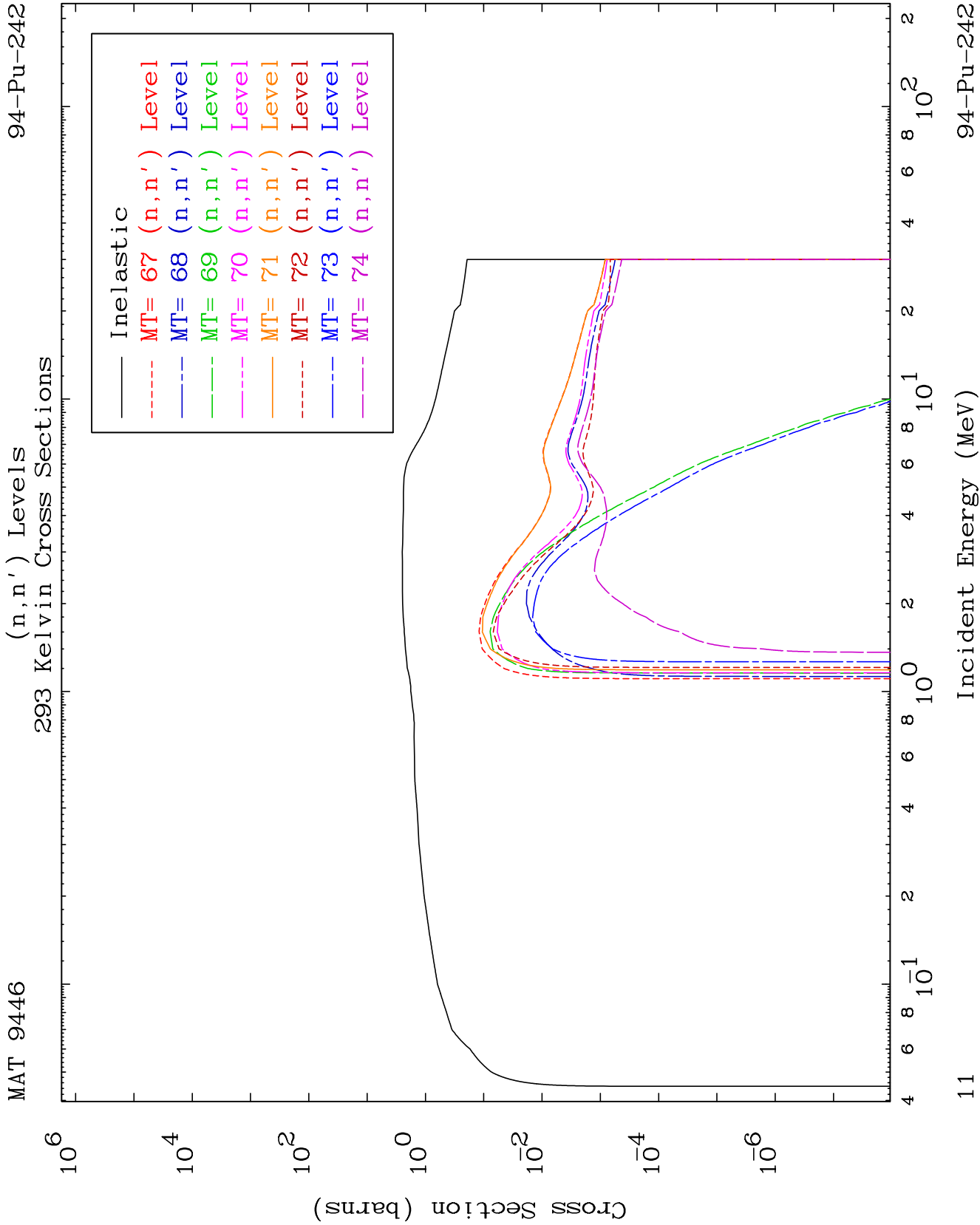
293 Kelvin Cross Sections

94-Pu-242





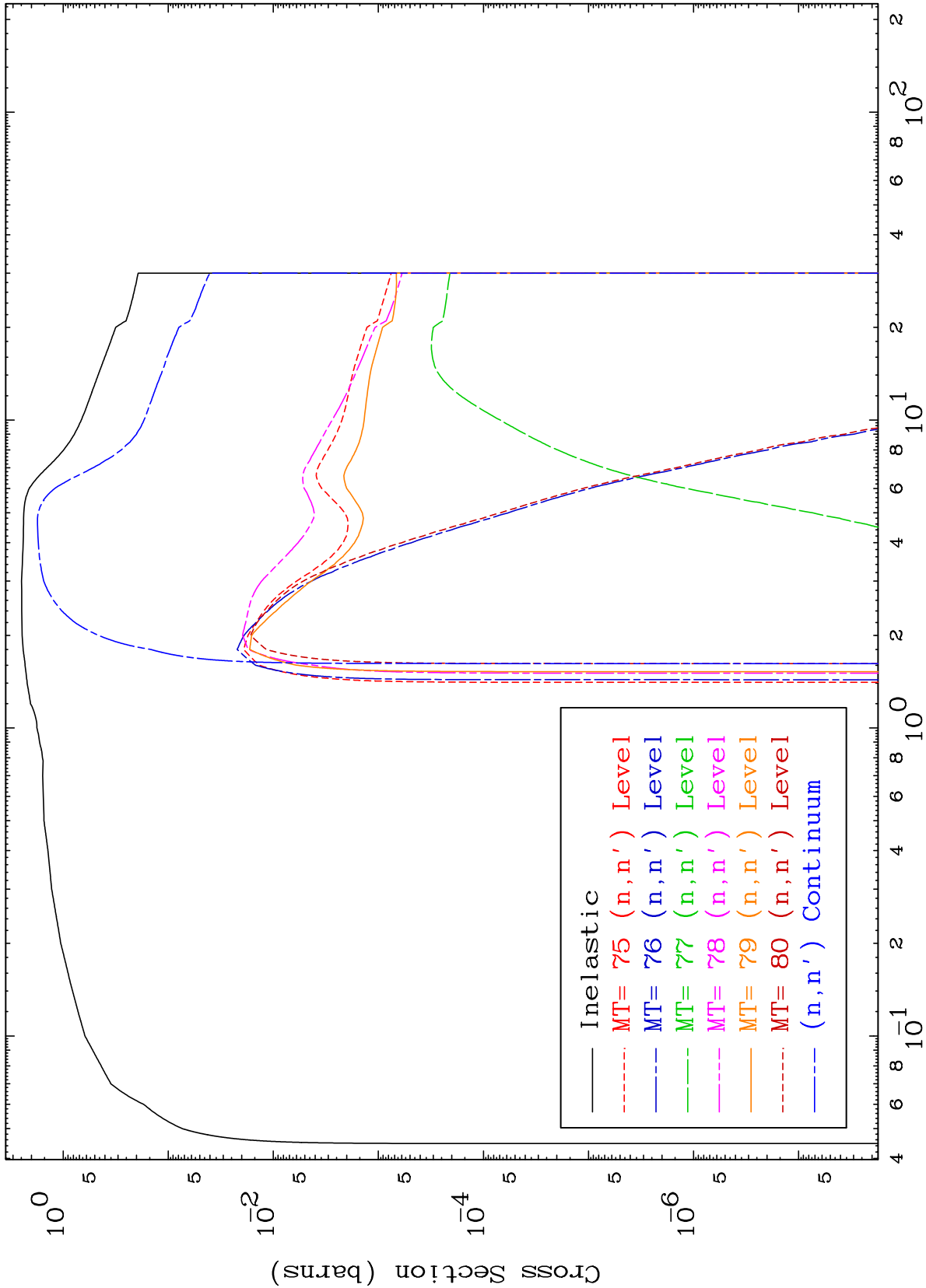




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(n,n') Levels
293 Kelvin Cross Sections

94-Pu-242



12

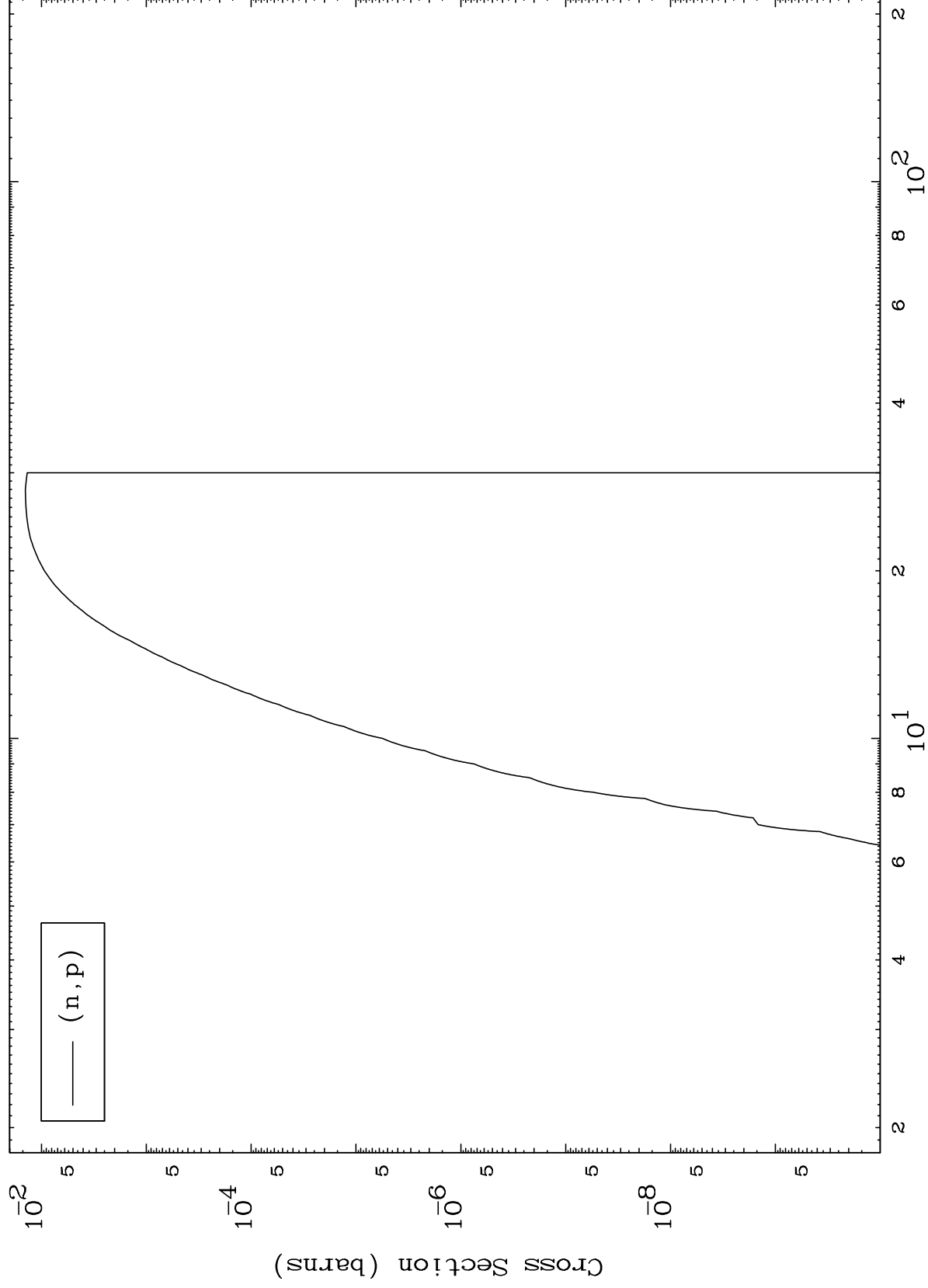
Incident Energy (MeV)

94-Pu-242

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94-Pu-242

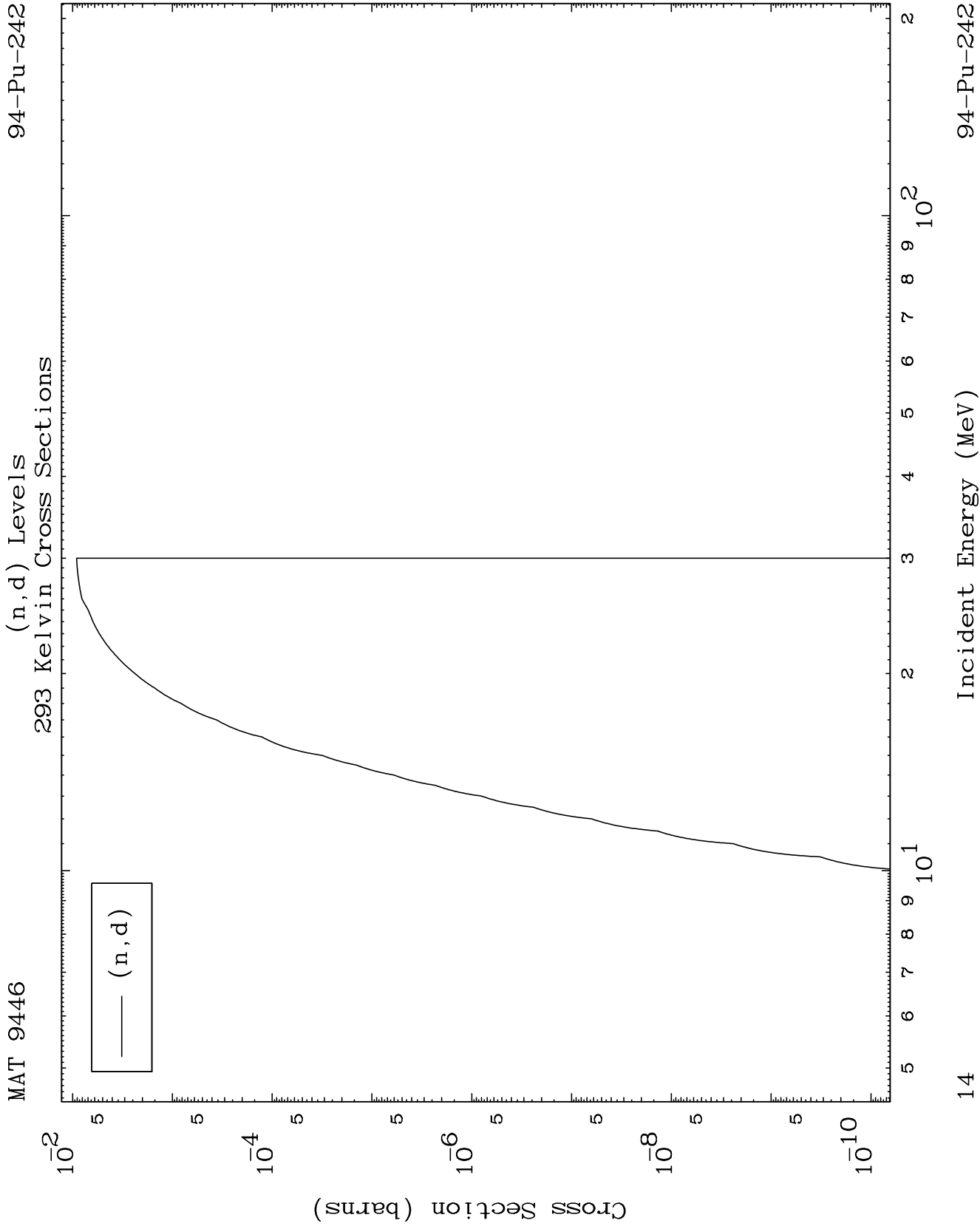
(n,p) Levels
293 Kelvin Cross Sections



94-Pu-242

Incident Energy (MeV)

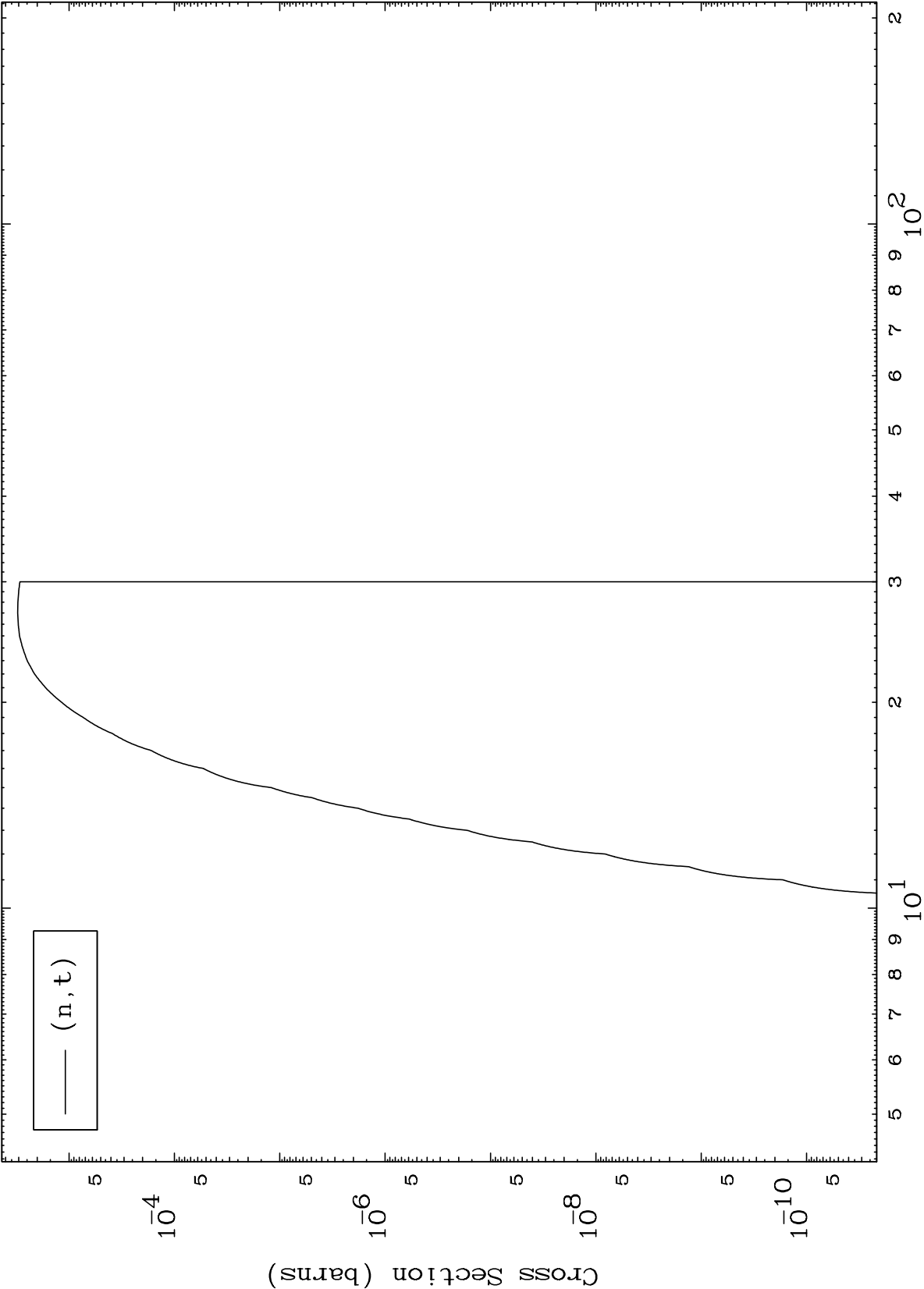
13



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94-Pu-242

(n,t) Levels
293 Kelvin Cross Sections



94-Pu-242

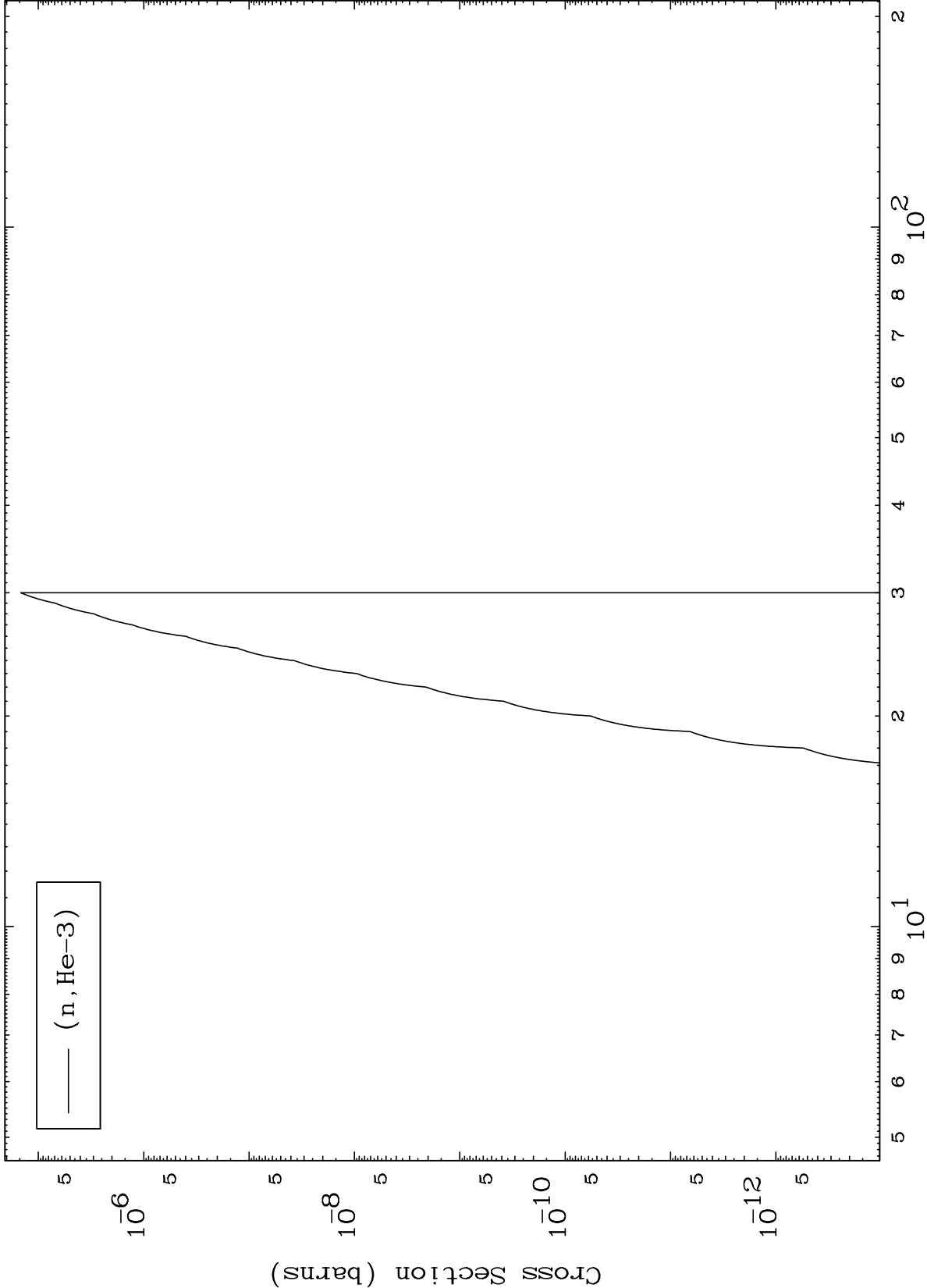
Incident Energy (MeV)

15

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(n,He3) Levels
293 Kelvin Cross Sections

94-Pu-242



16

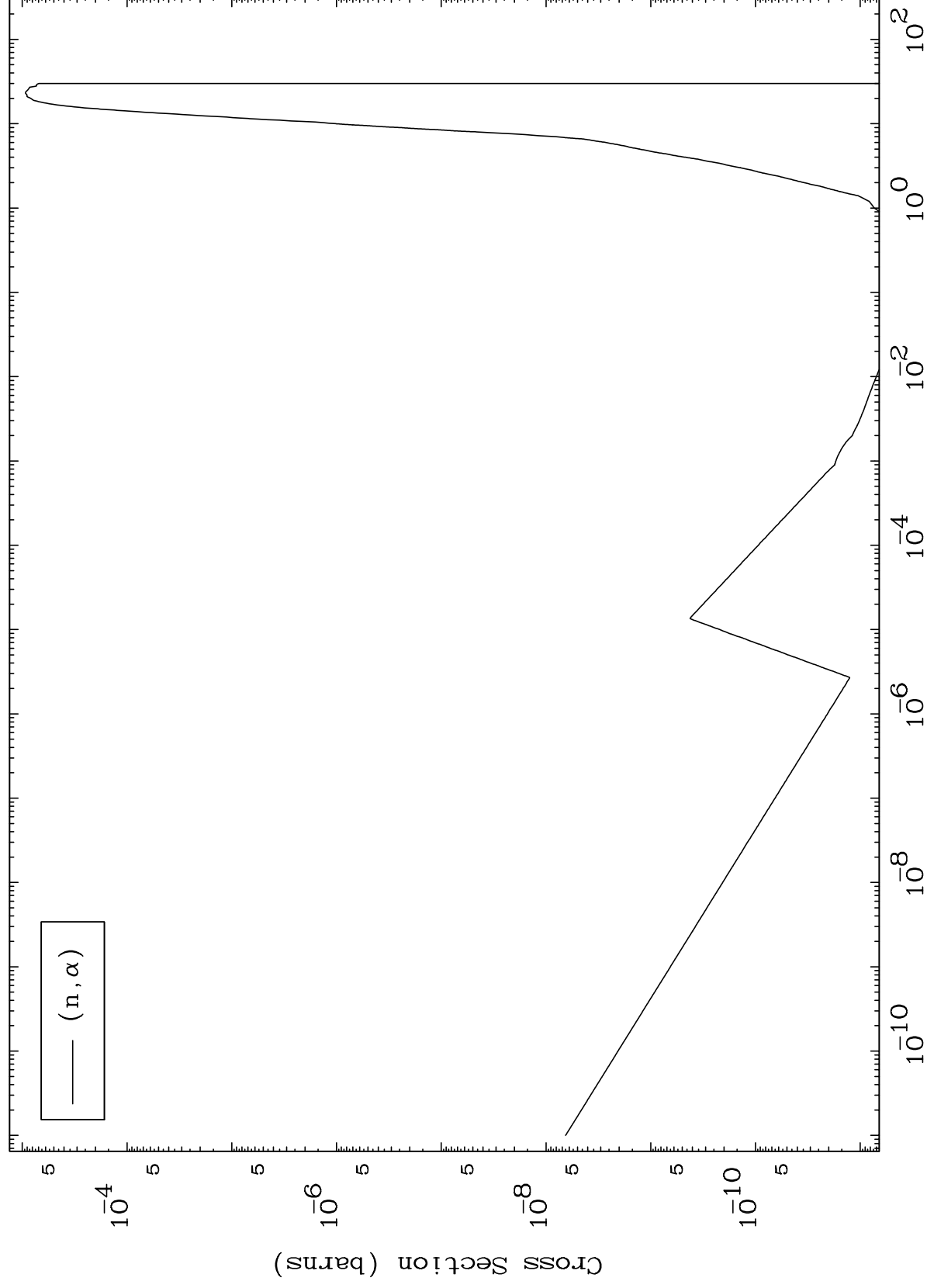
Incident Energy (MeV)

94-Pu-242

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(n, α) Levels
293 Kelvin Cross Sections

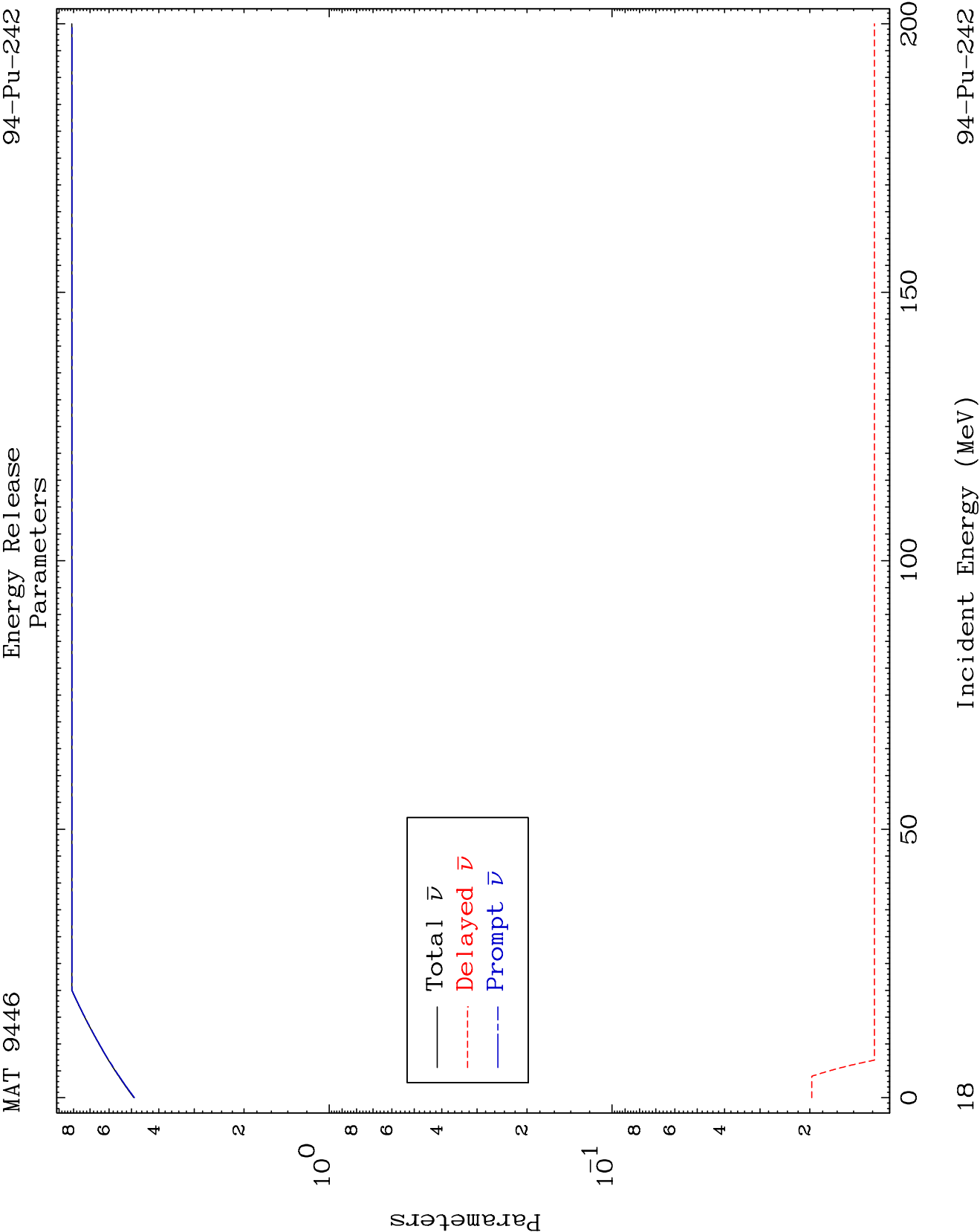
94-Pu-242

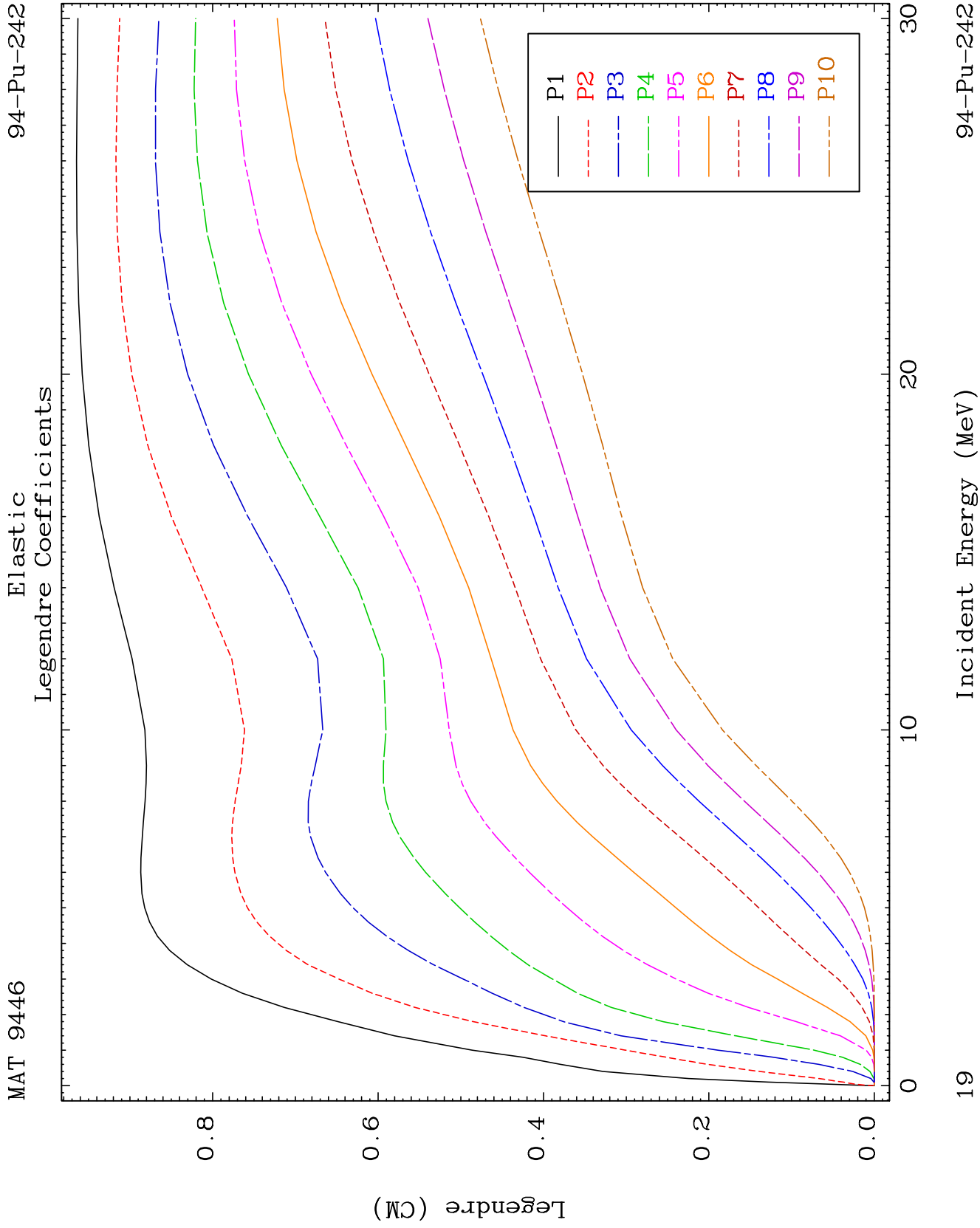


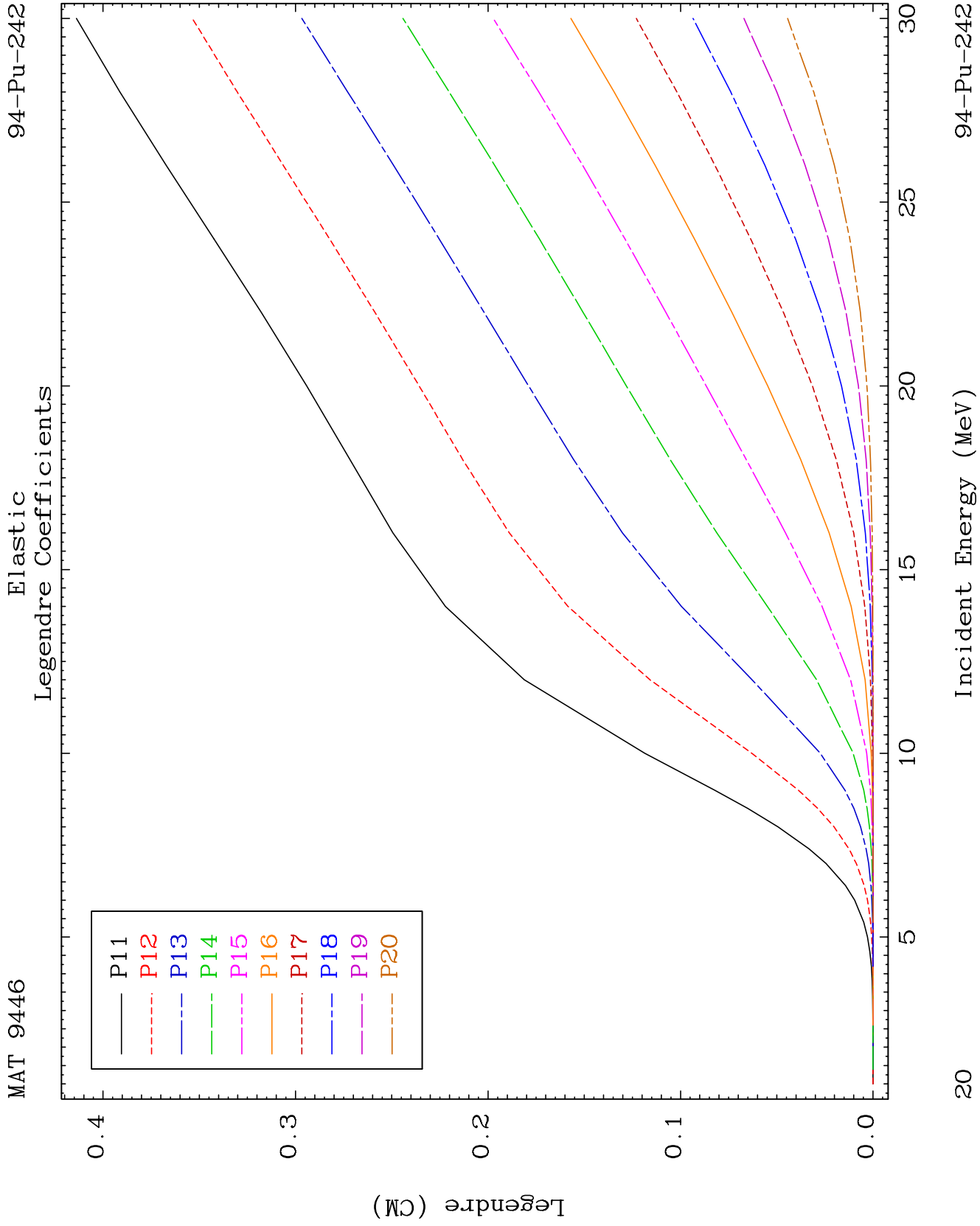
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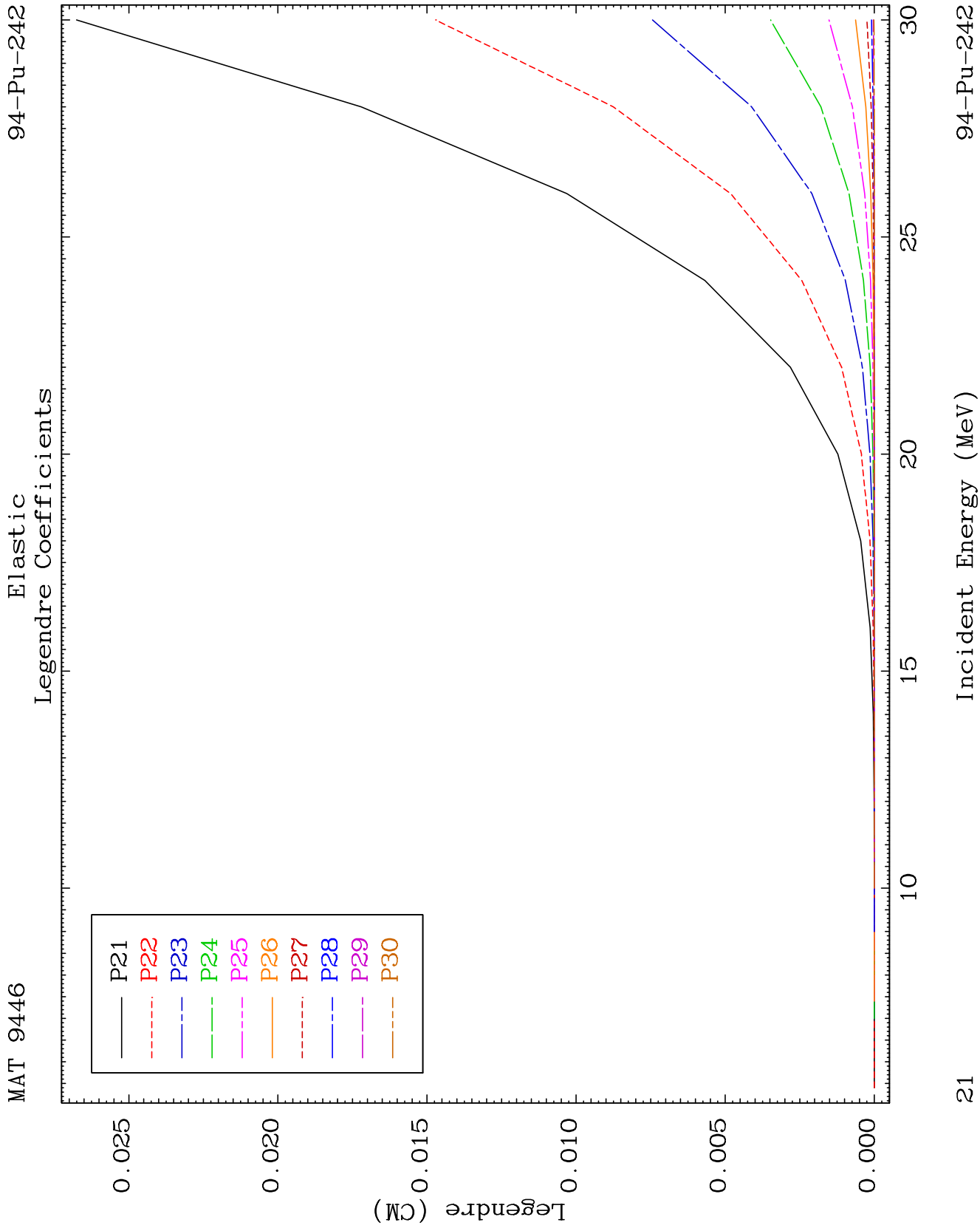
Incident Energy (MeV)

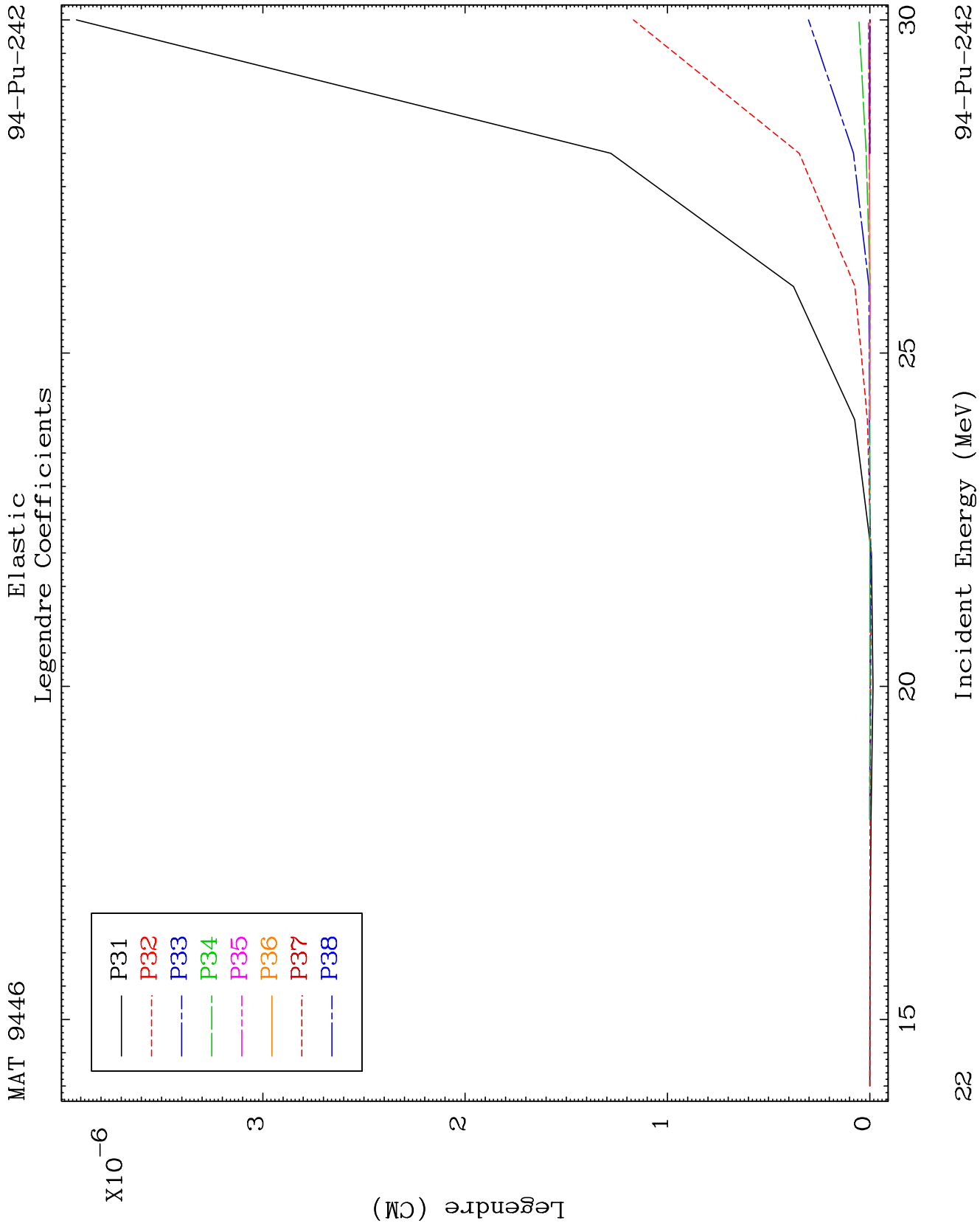
94-Pu-242

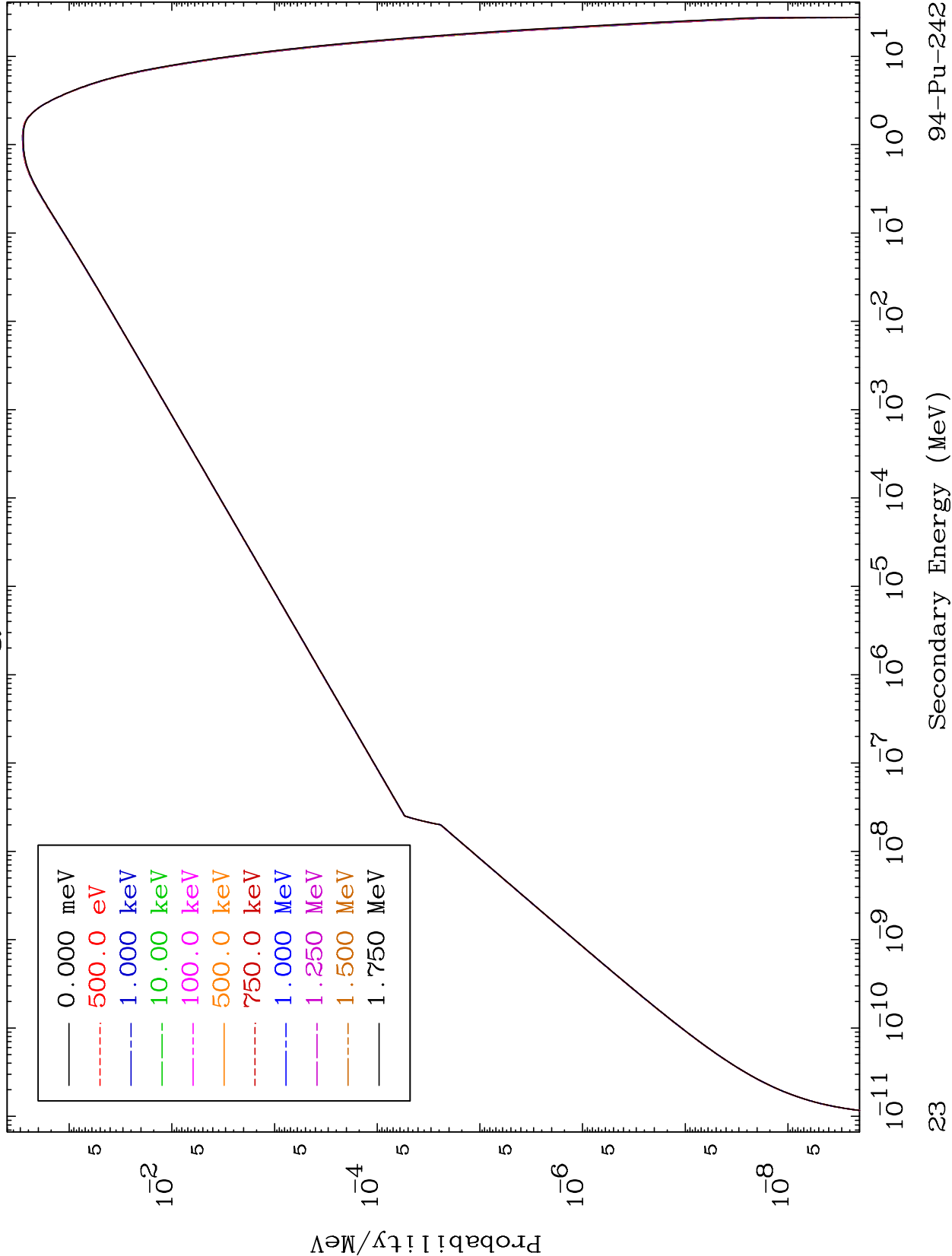








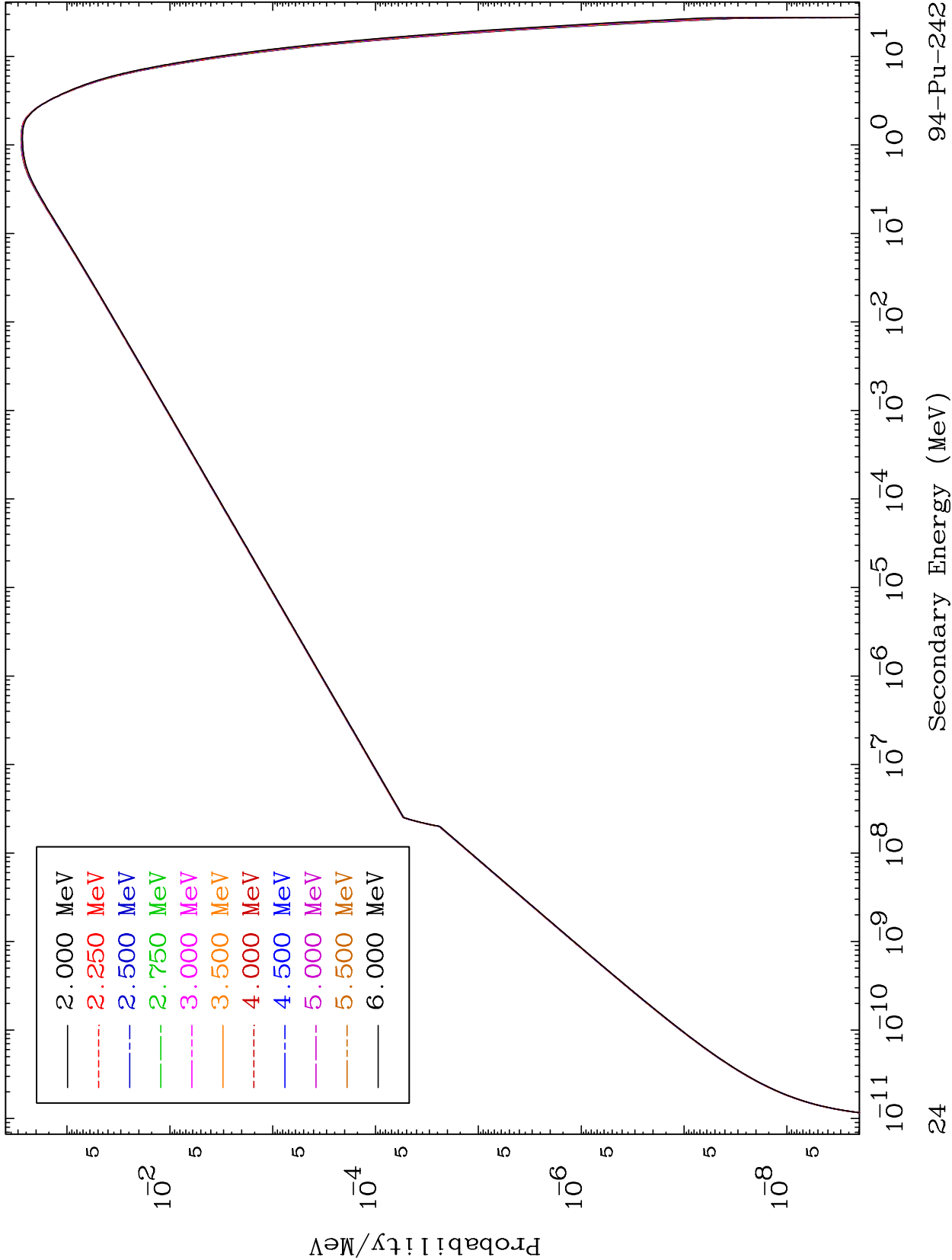


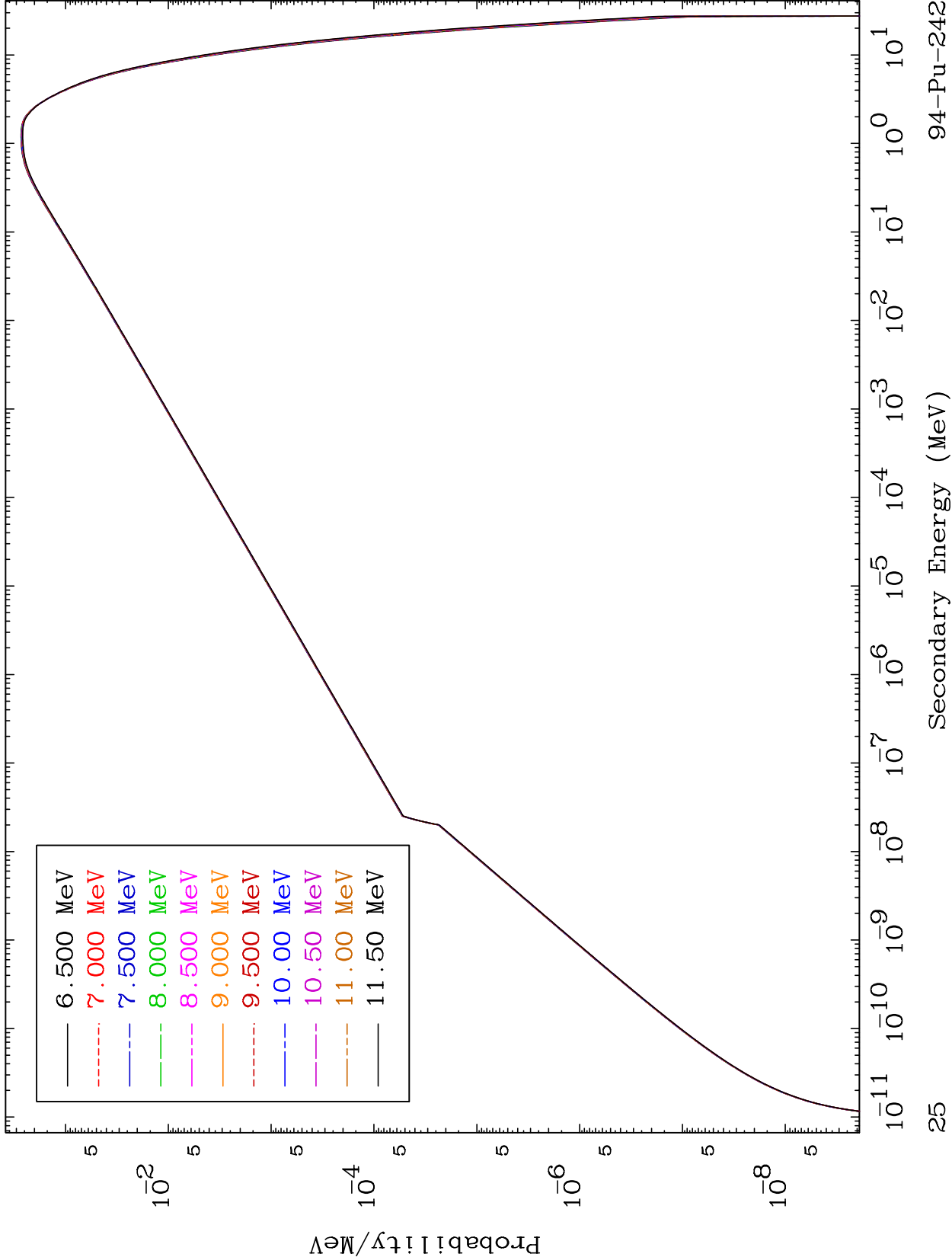


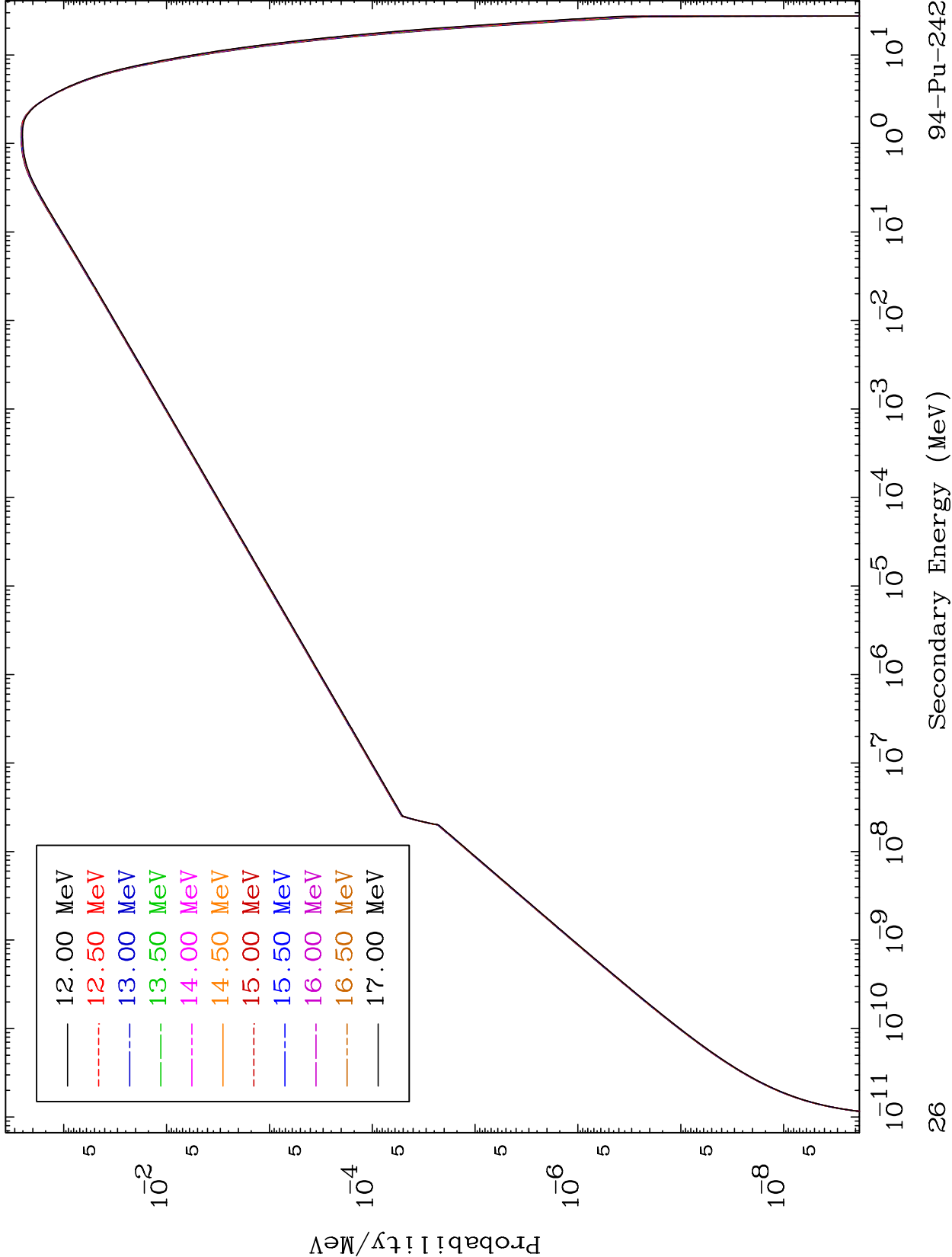
MAT 9446

Fission
Energy Distributions

94-Pu-242



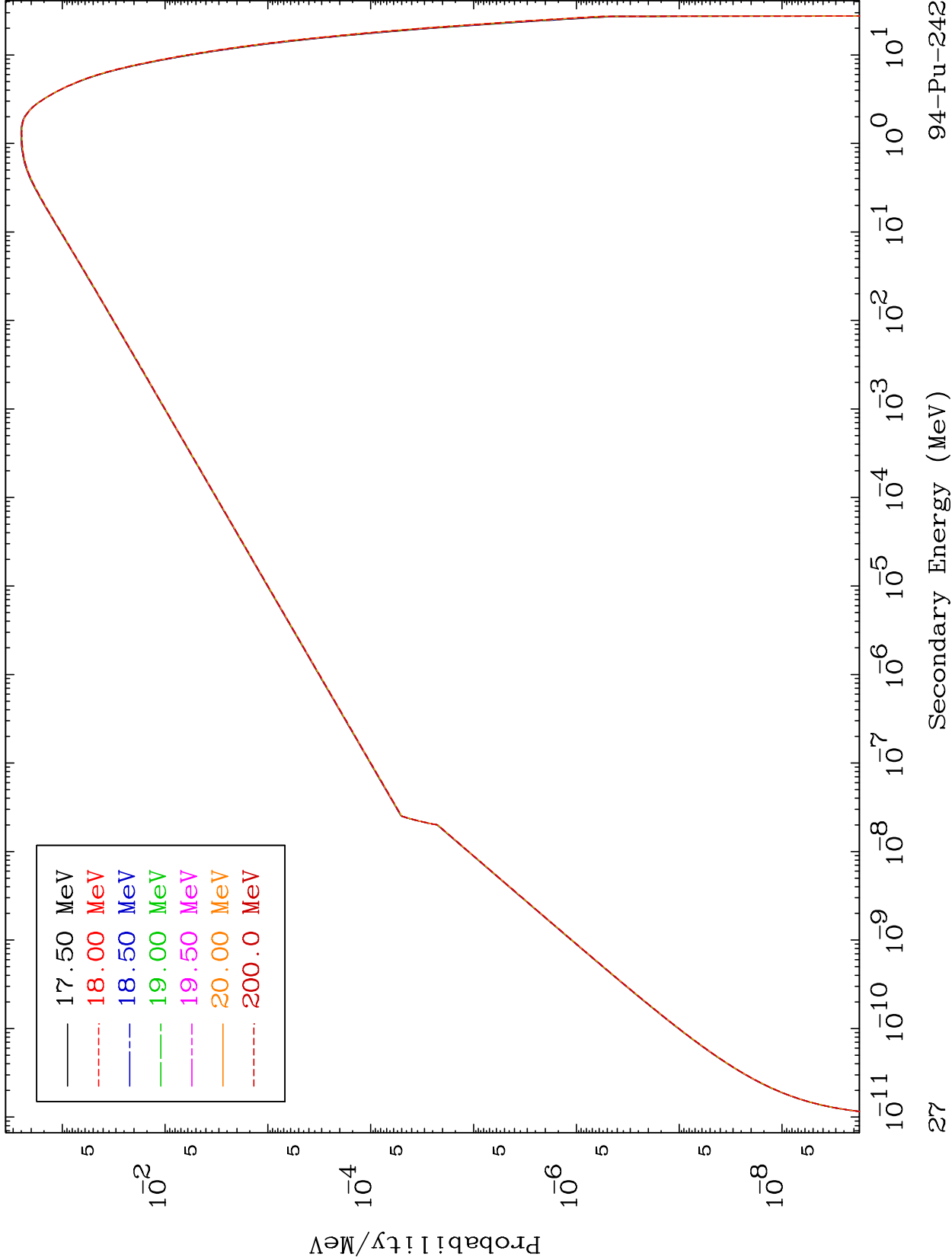




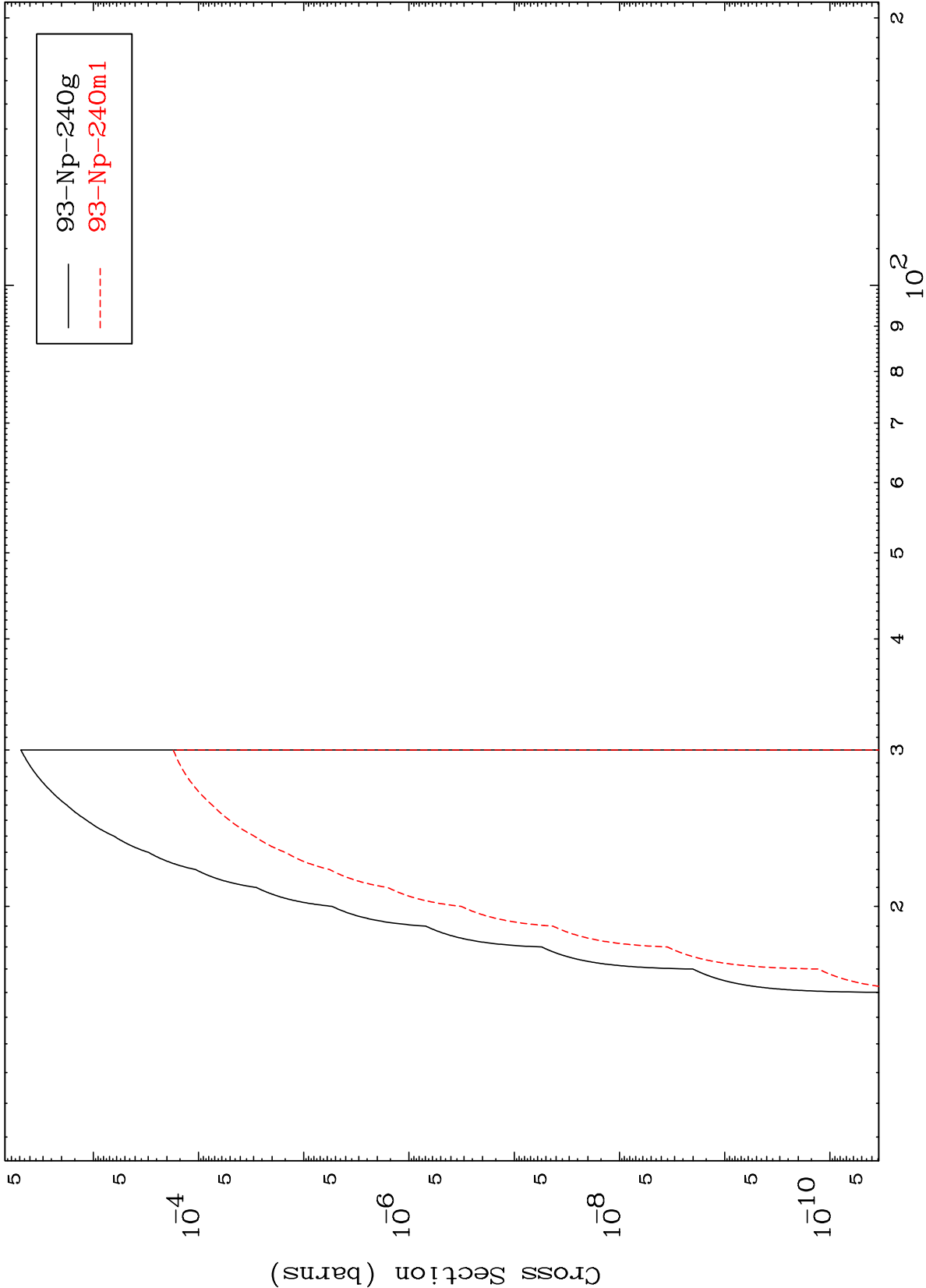
MAT 9446

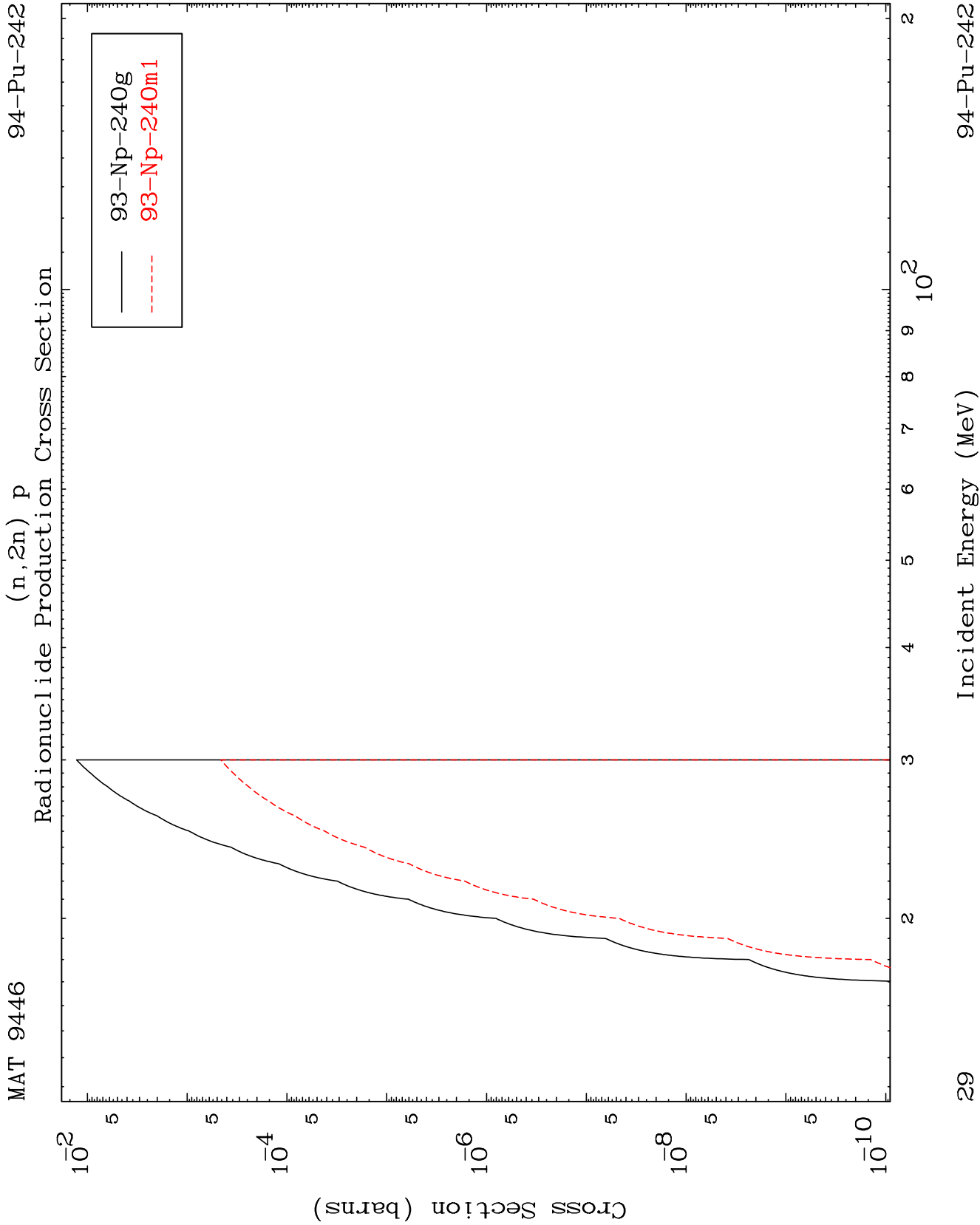
Fission
Energy Distributions

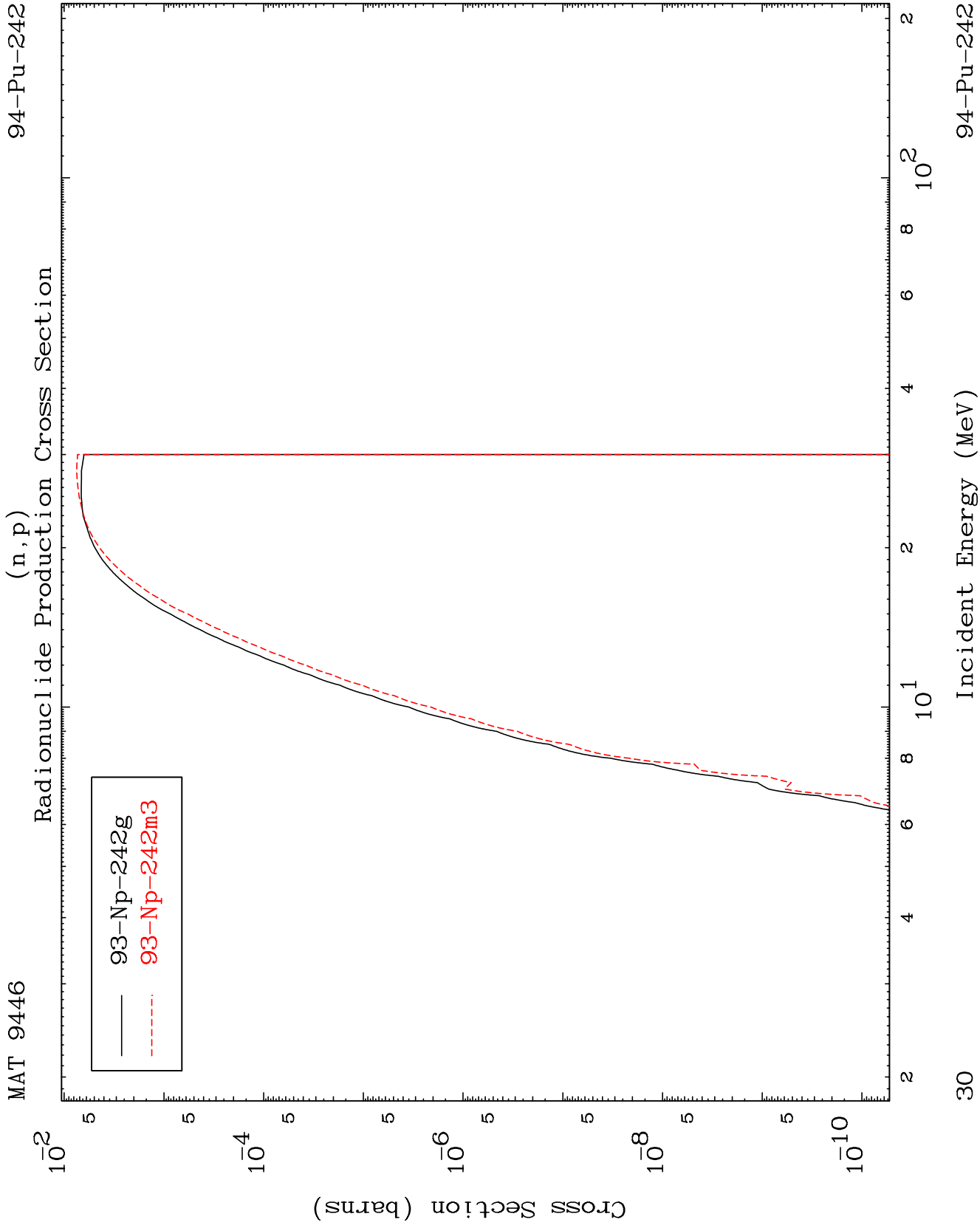
94-Pu-242



(n,n') d
Radionuclide Production Cross Section



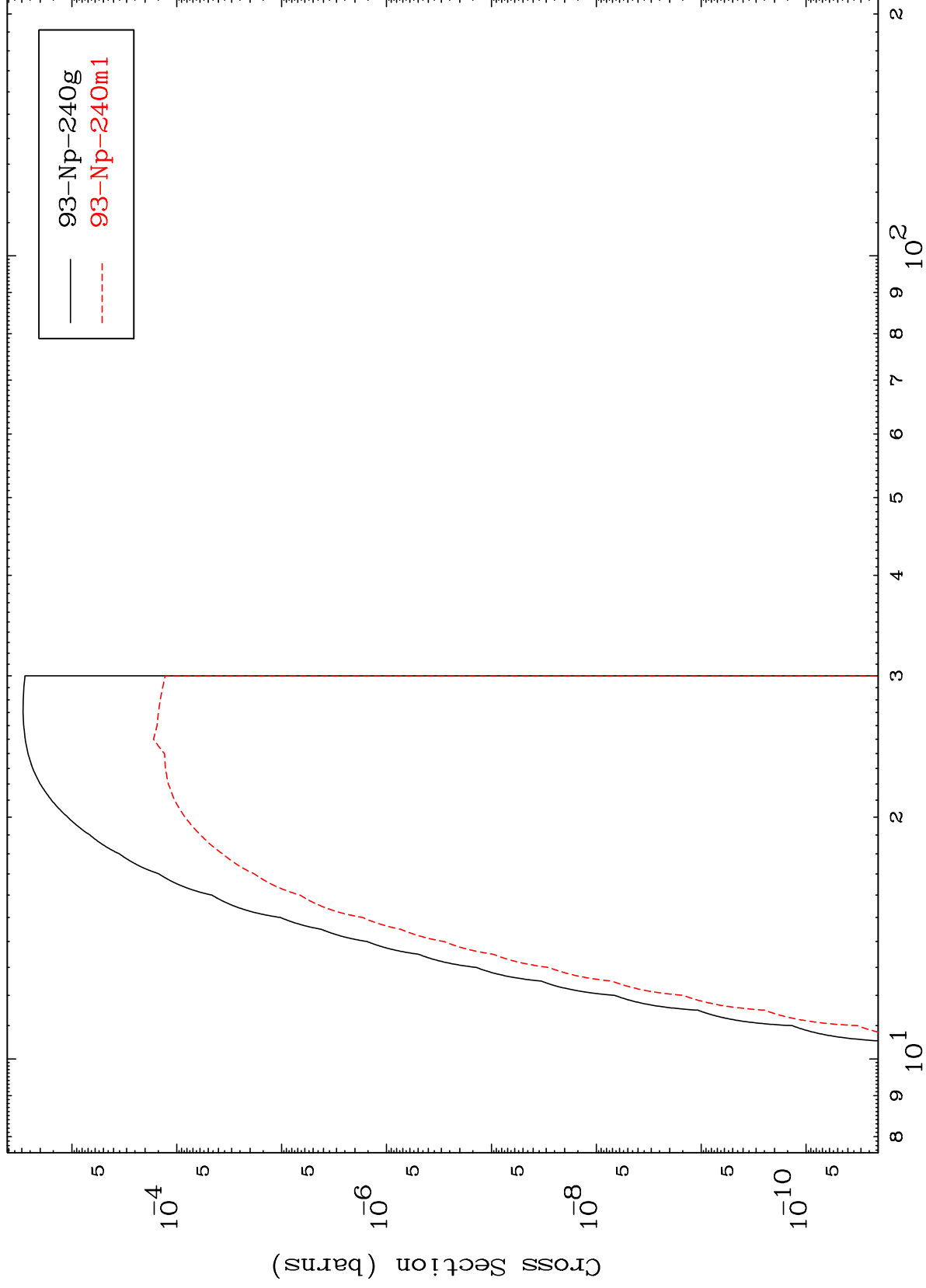




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94-Pu-242

(n,t)
Radionuclide Production Cross Section



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Incident Energy (MeV)

94-Pu-242