

Program EVALPLOT  
(Version 2021-1)

by

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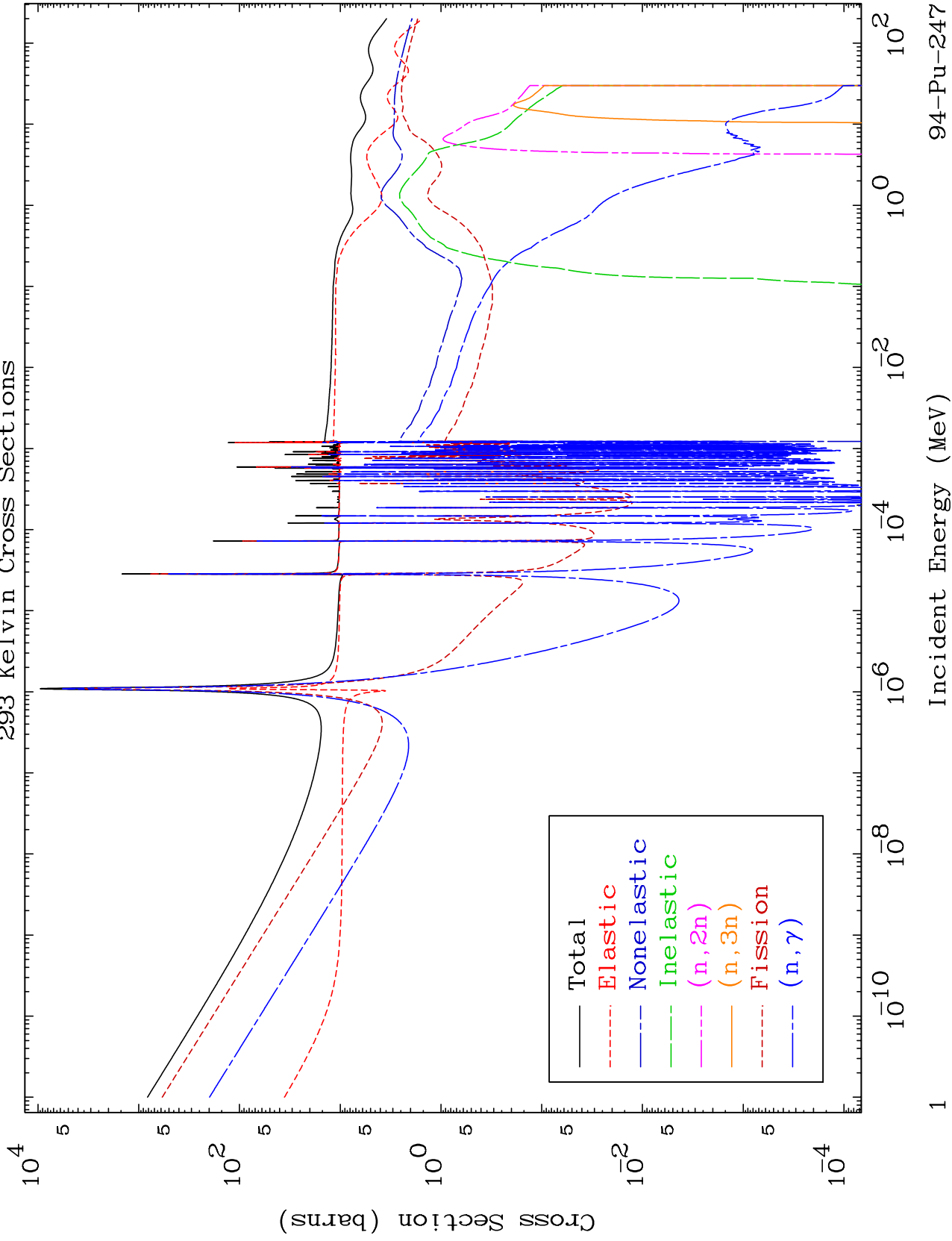
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Press Mouse Button to Start

MAT 9461

Neutron Major  
293 Kelvin Cross Sections

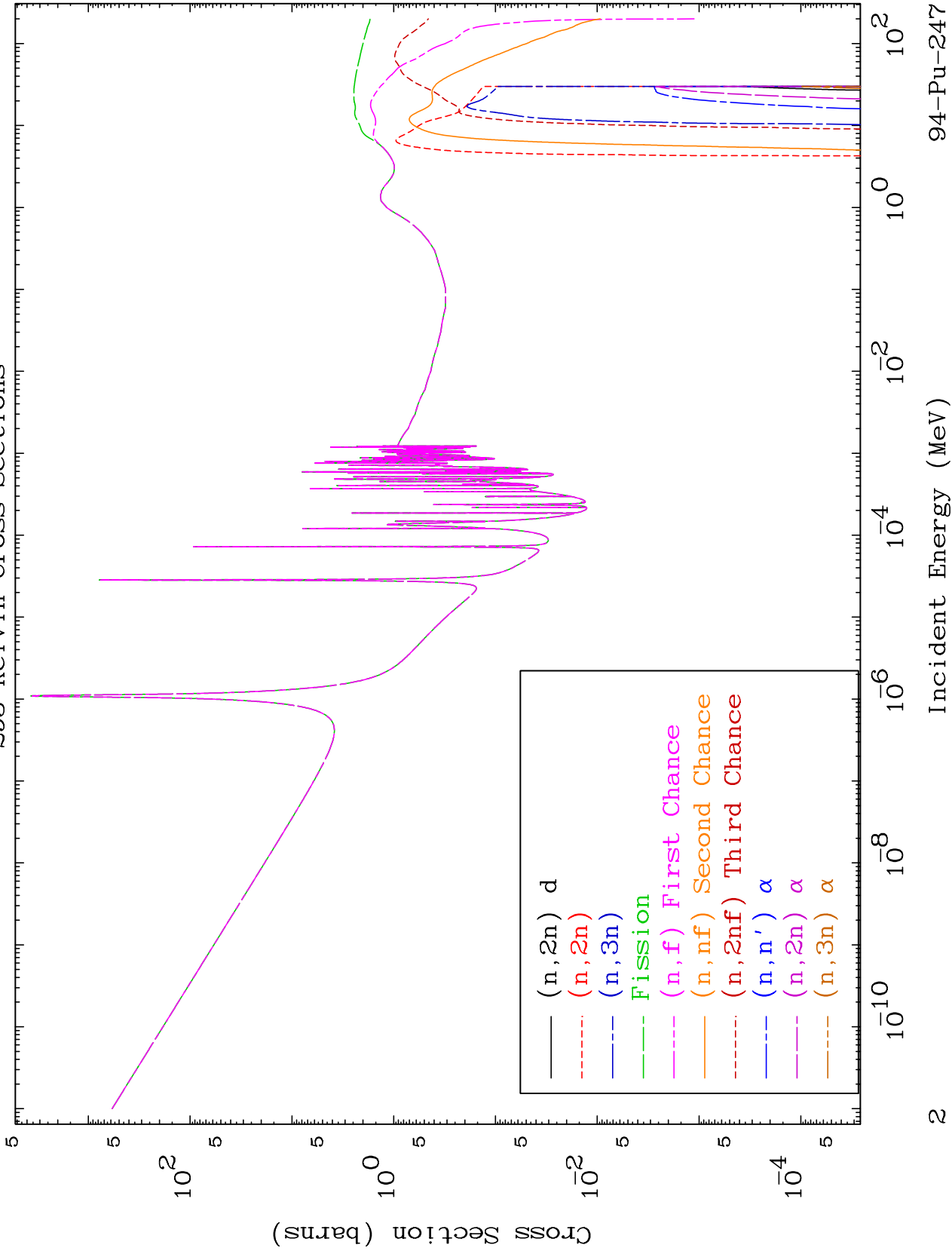
94-Pu-247



MAT 9461

Neutron Absorption  
293 Kelvin Cross Sections

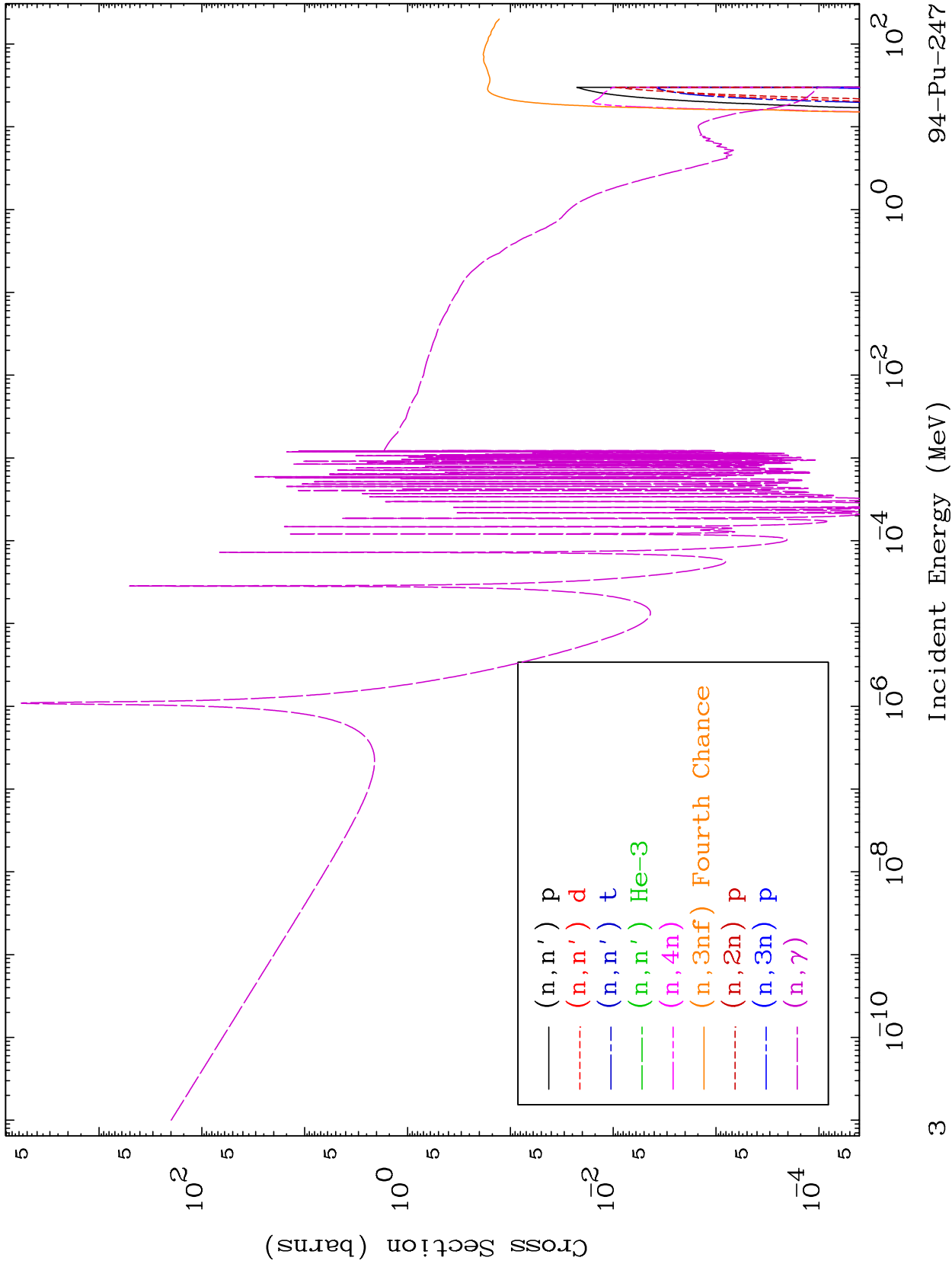
94-Pu-247



MAT 9461

Neutron Absorption  
293 Kelvin Cross Sections

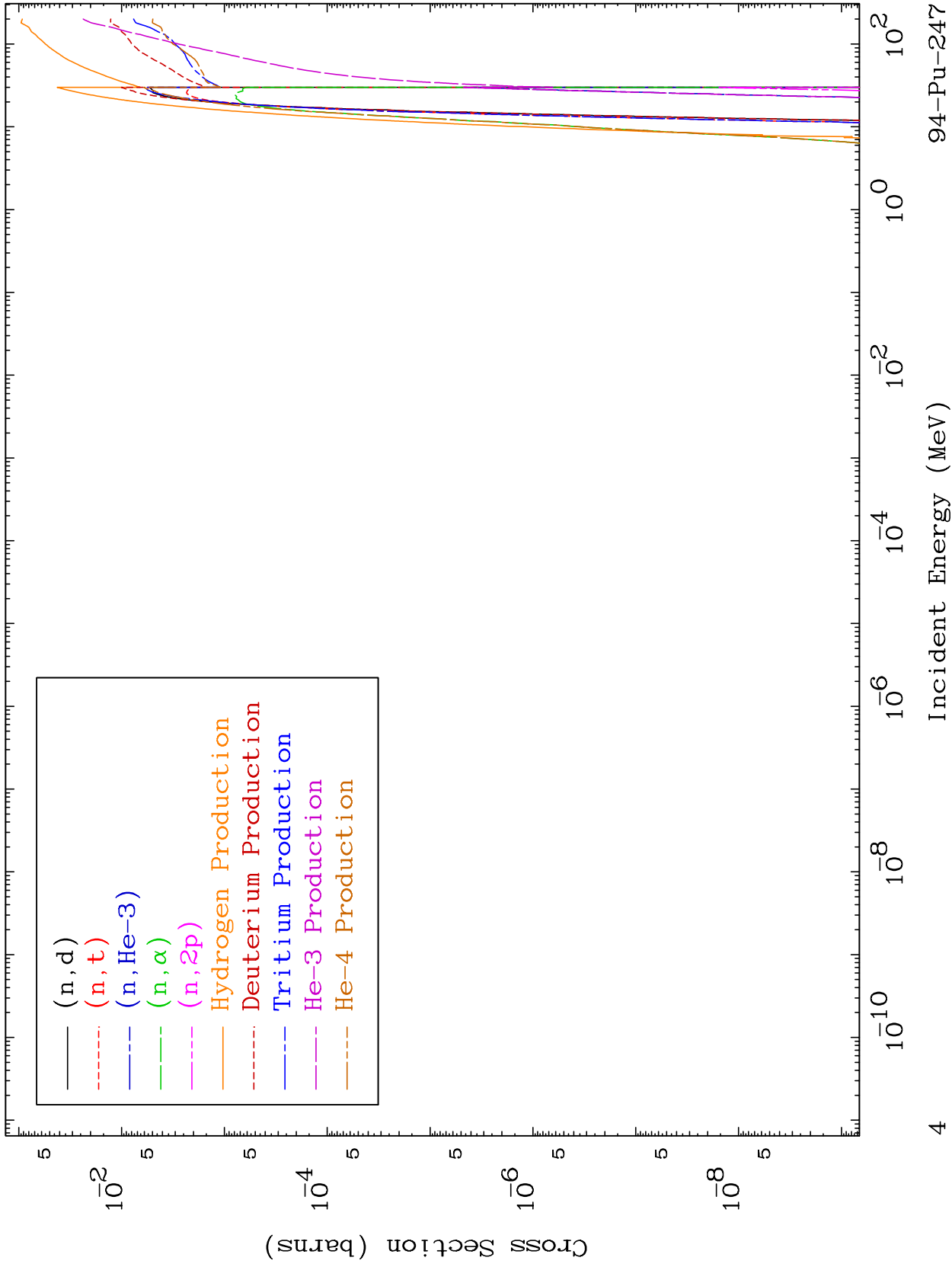
94-Pu-247



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Neutron Absorption  
293 Kelvin Cross Sections

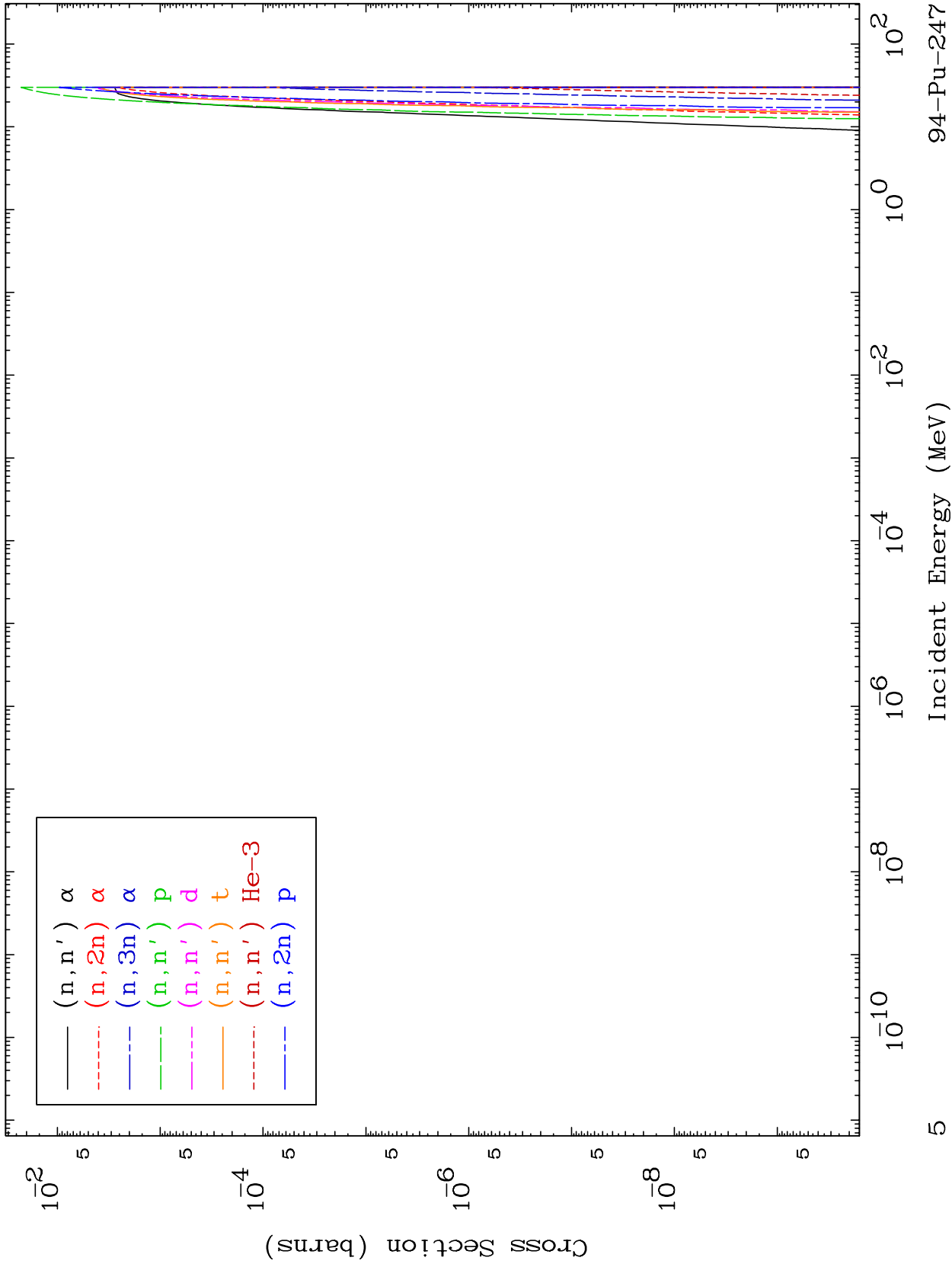
94-Pu-247



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Charged Particle  
293 Kelvin Cross Sections

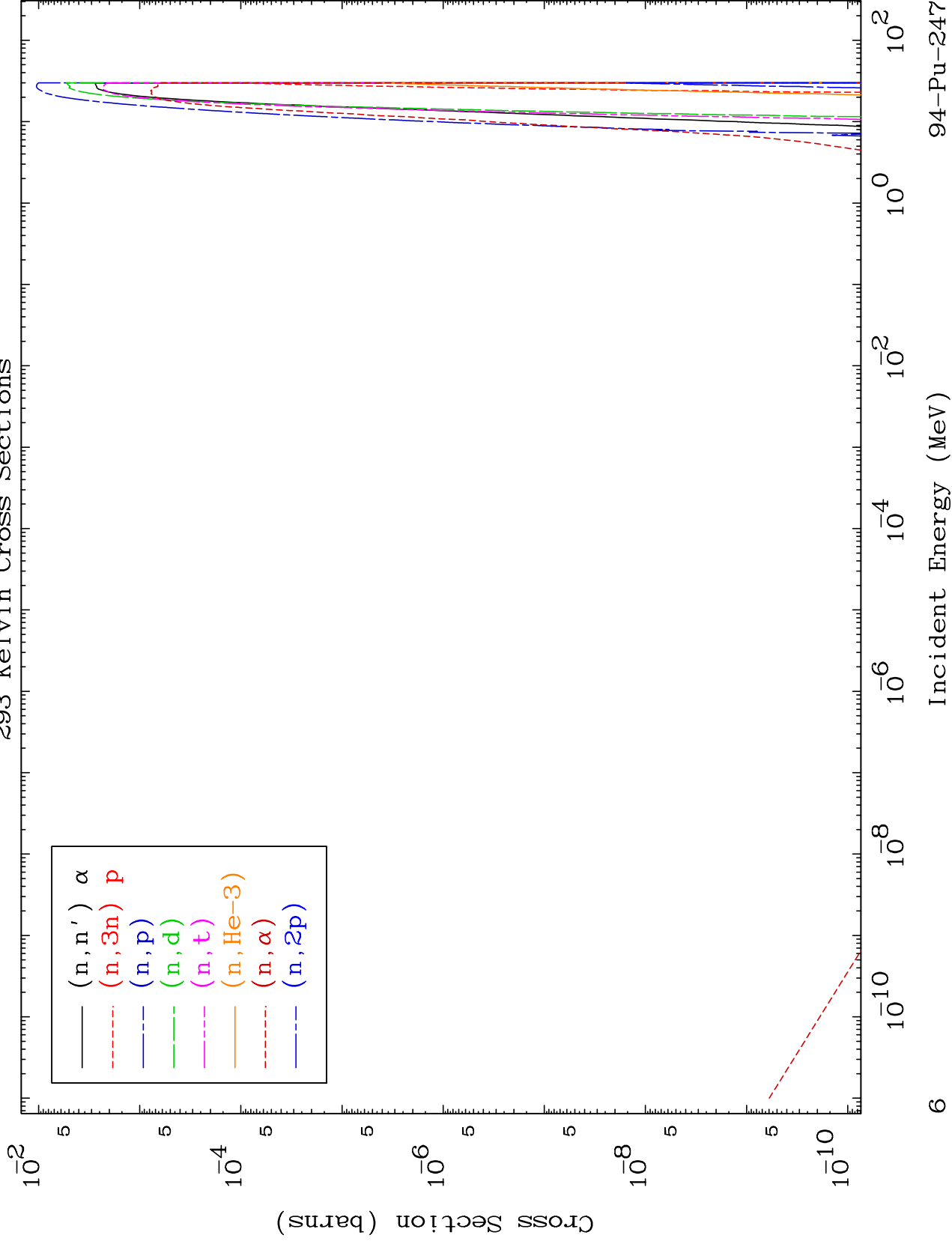
94-Pu-247



MAT 9461

Charged Particle  
293 Kelvin Cross Sections

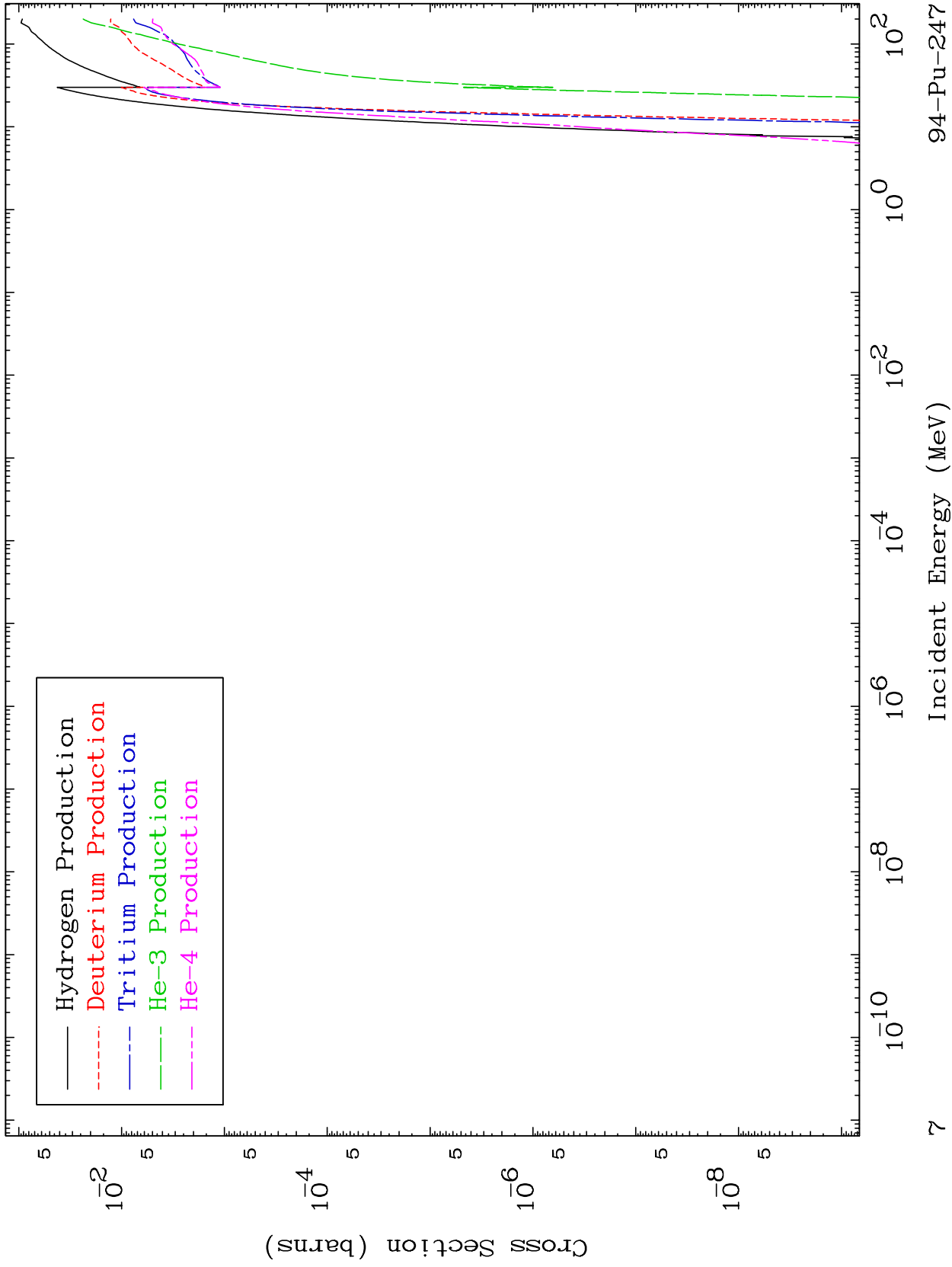
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Particle Production  
293 Kelvin Cross Sections

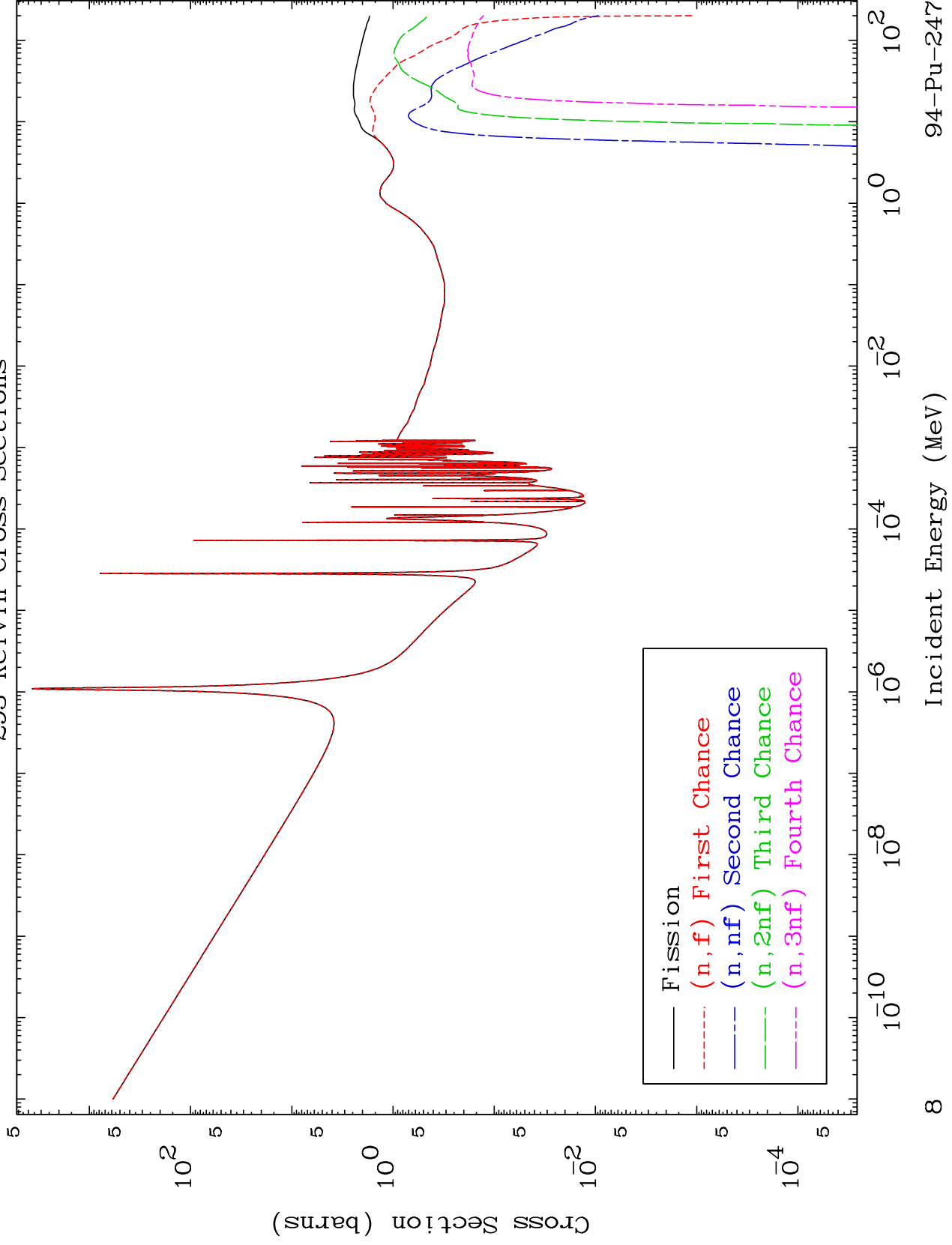
94-Pu-247



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Fission  
293 Kelvin Cross Sections

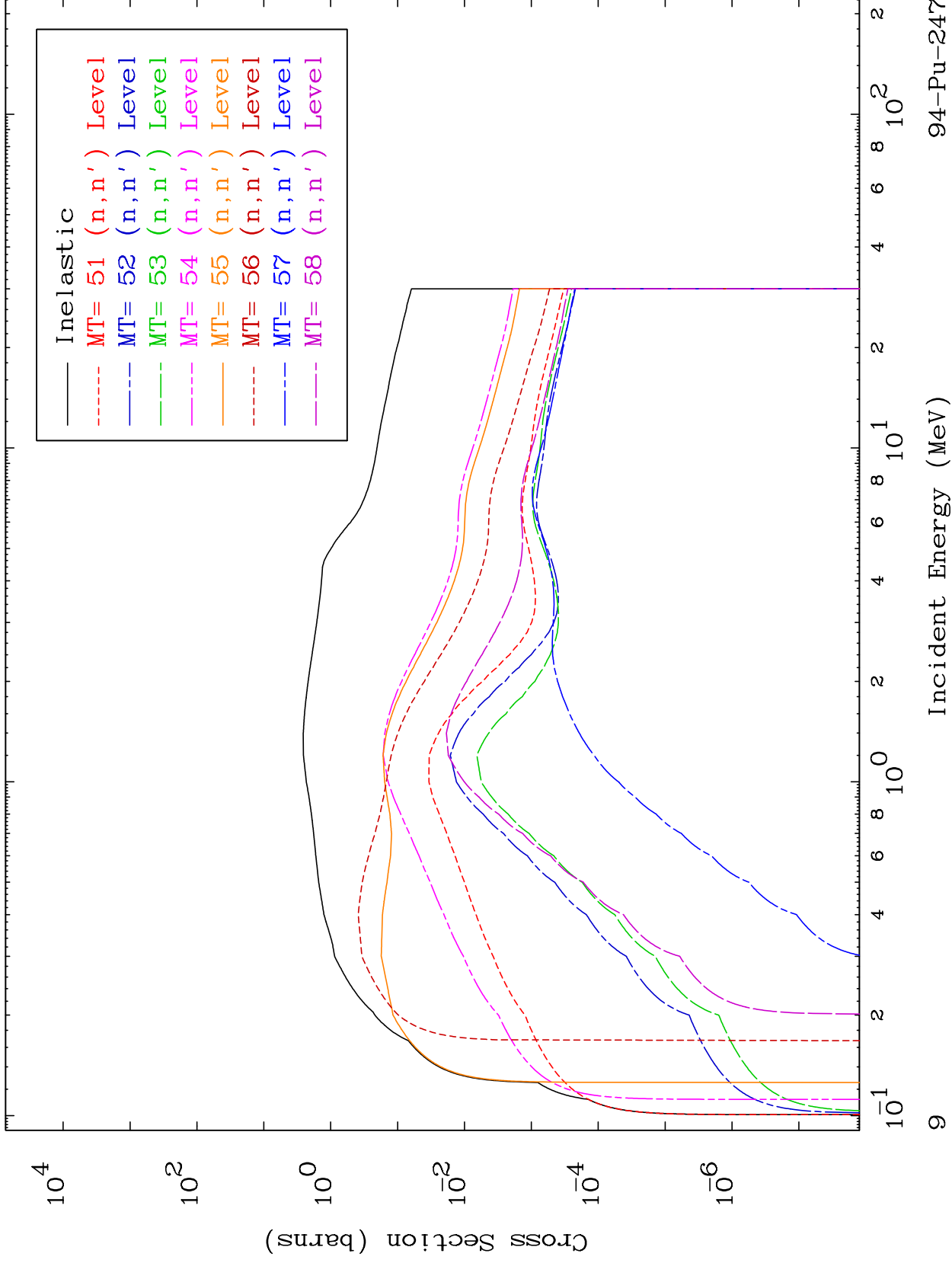
94-Pu-247



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(n,n') Levels  
293 Kelvin Cross Sections

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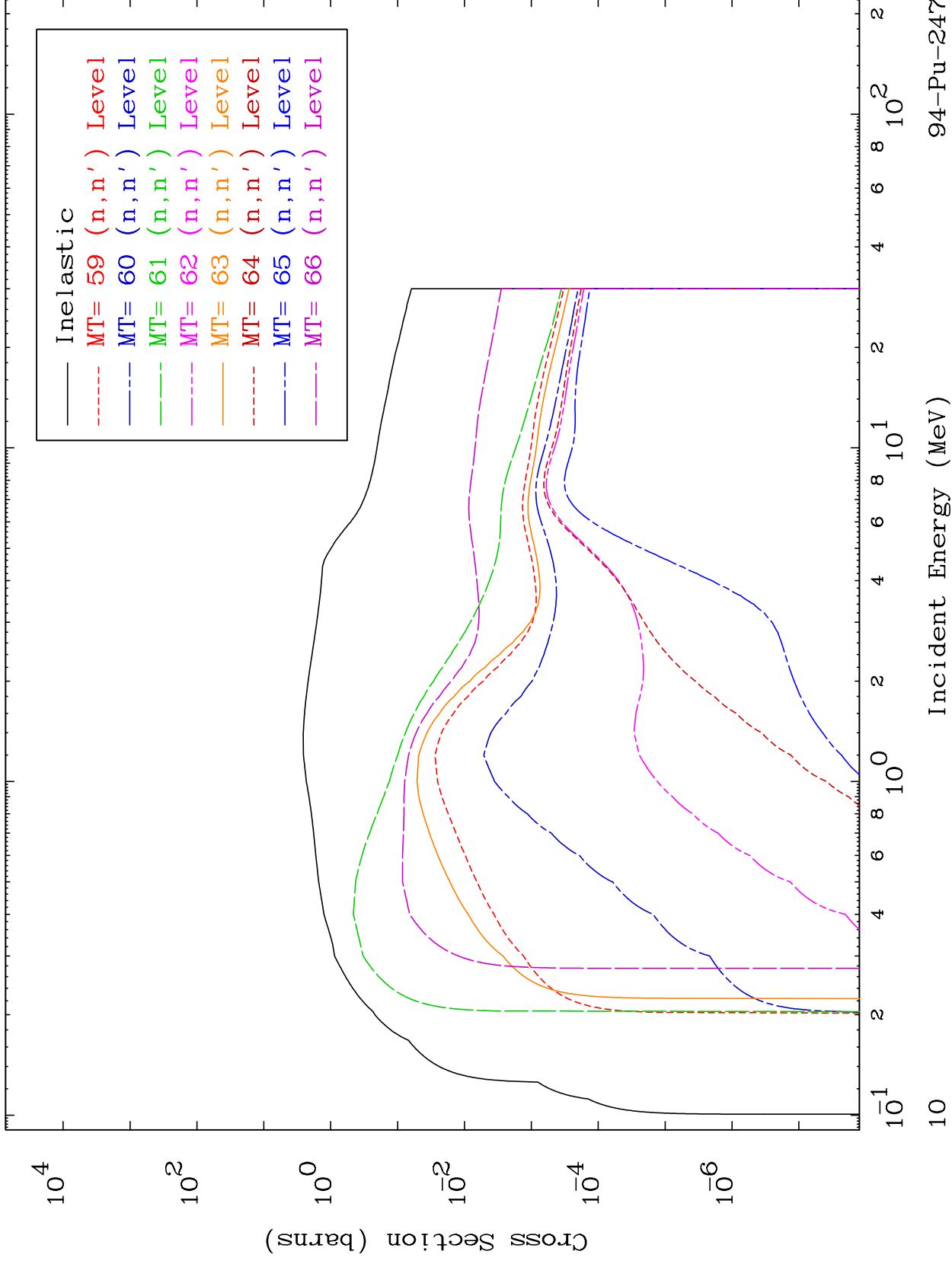


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(n,n') Levels  
293 Kelvin Cross Sections

94-Pu-247



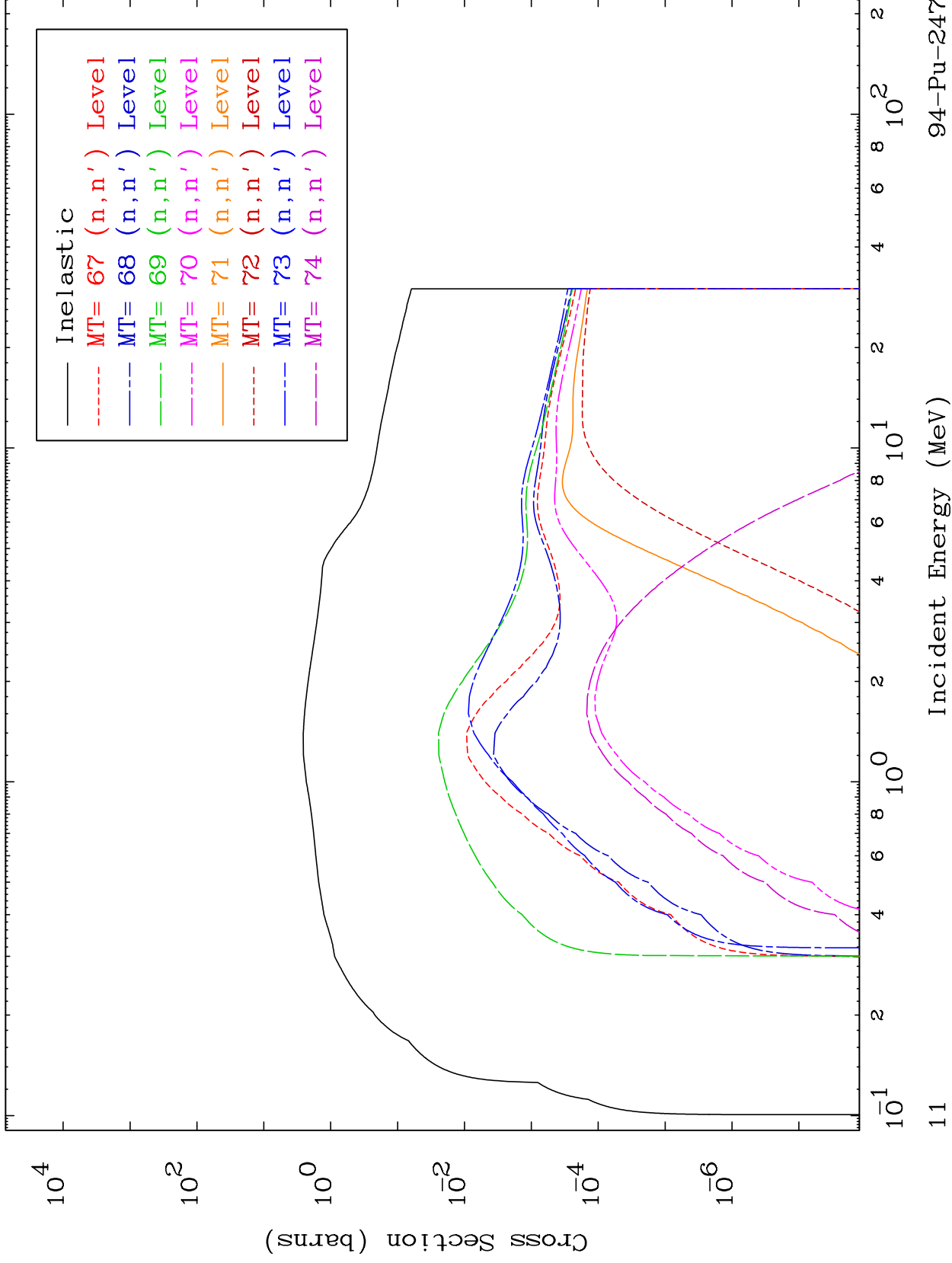
94-Pu-247

Incident Energy (MeV)

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(n,n') Levels  
293 Kelvin Cross Sections

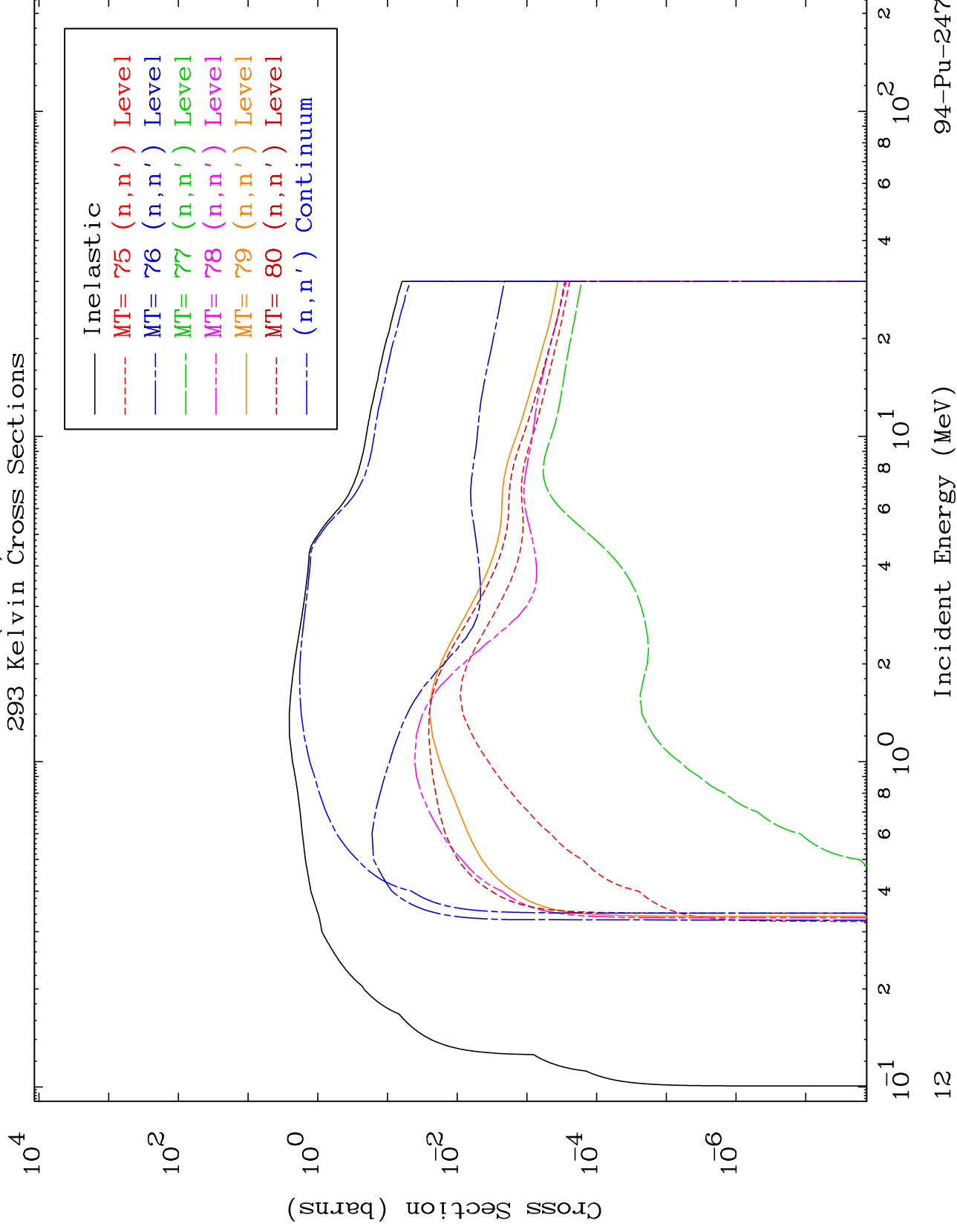
94-Pu-247



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(n,n') Levels  
293 Kelvin Cross Sections

94-Pu-247

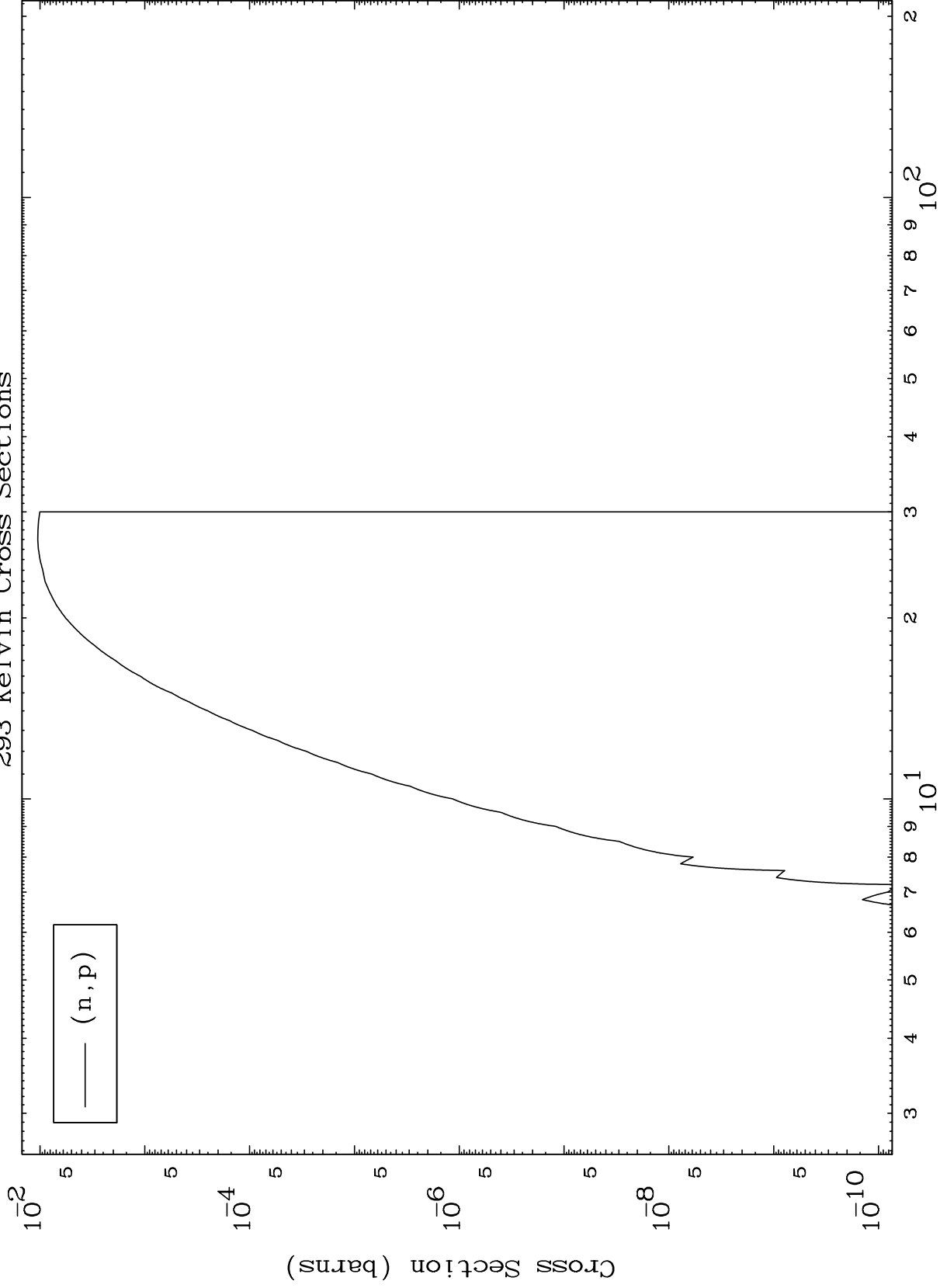


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(n,p) Levels  
293 Kelvin Cross Sections

94-Pu-247



13

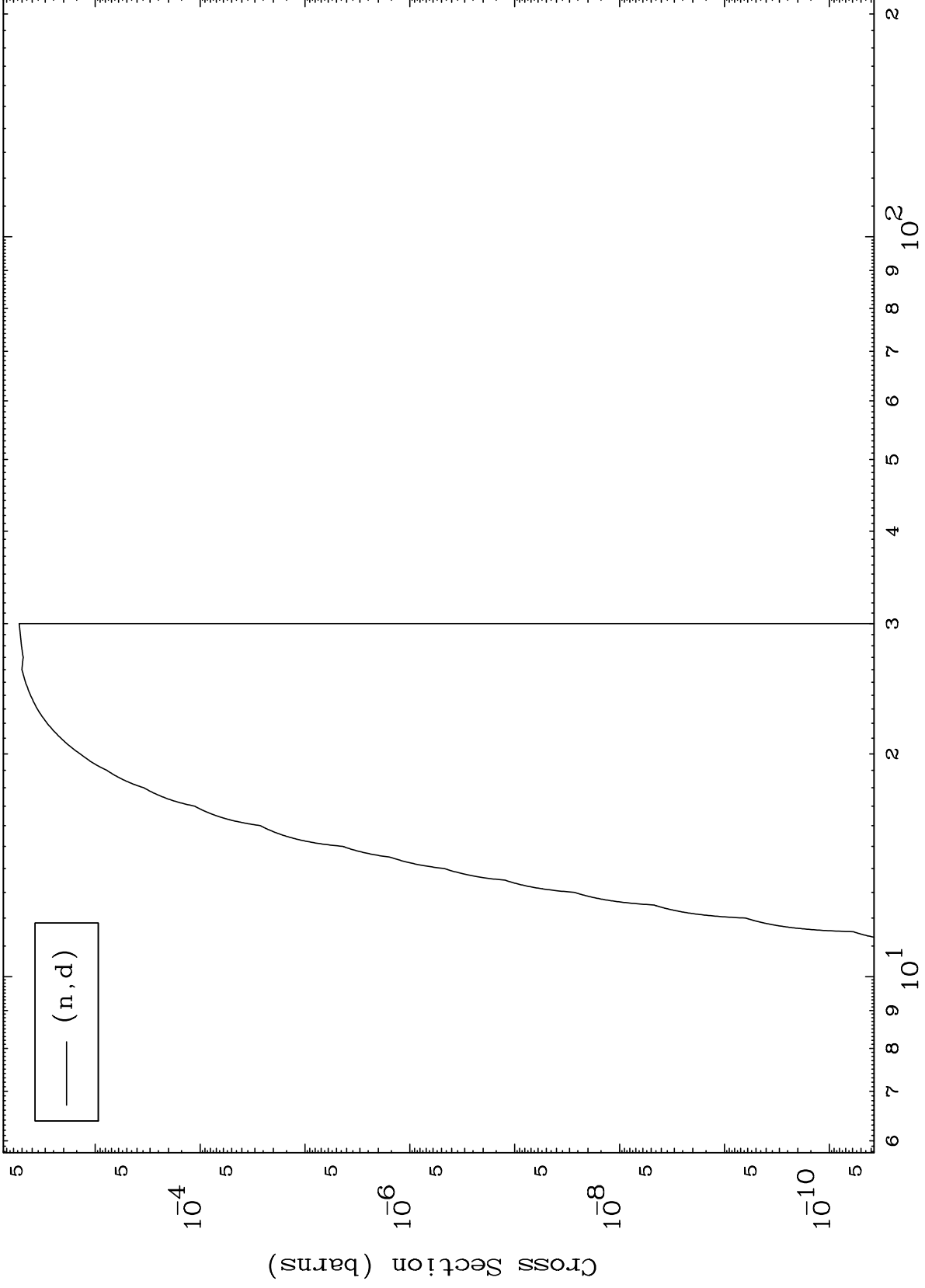
Incident Energy (MeV)

94-Pu-247

MAT 9461

(n,d) Levels  
293 Kelvin Cross Sections

94-Pu-247



14

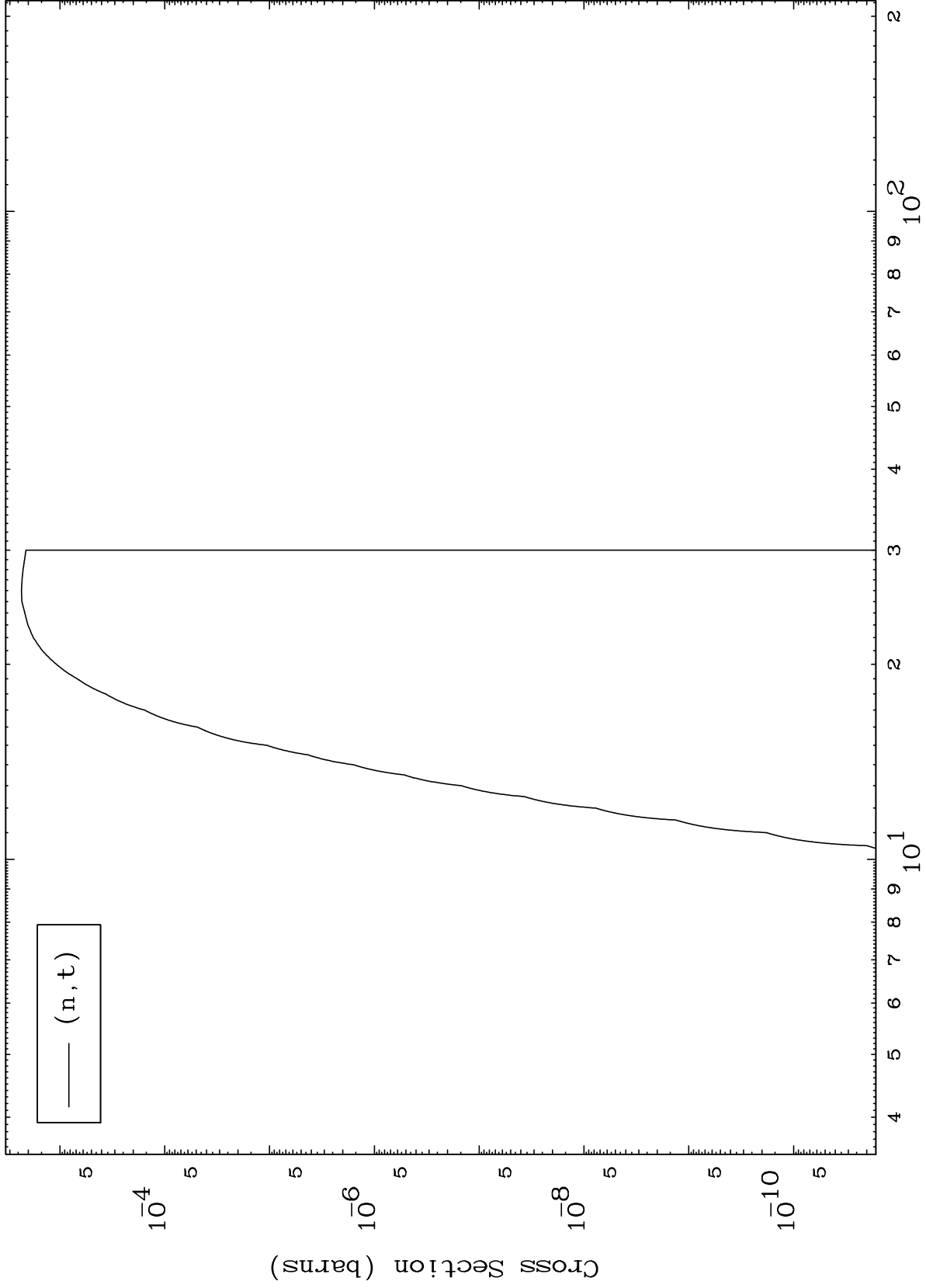
Incident Energy (MeV)

94-Pu-247

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(n,t) Levels  
293 Kelvin Cross Sections

94-Pu-247



15

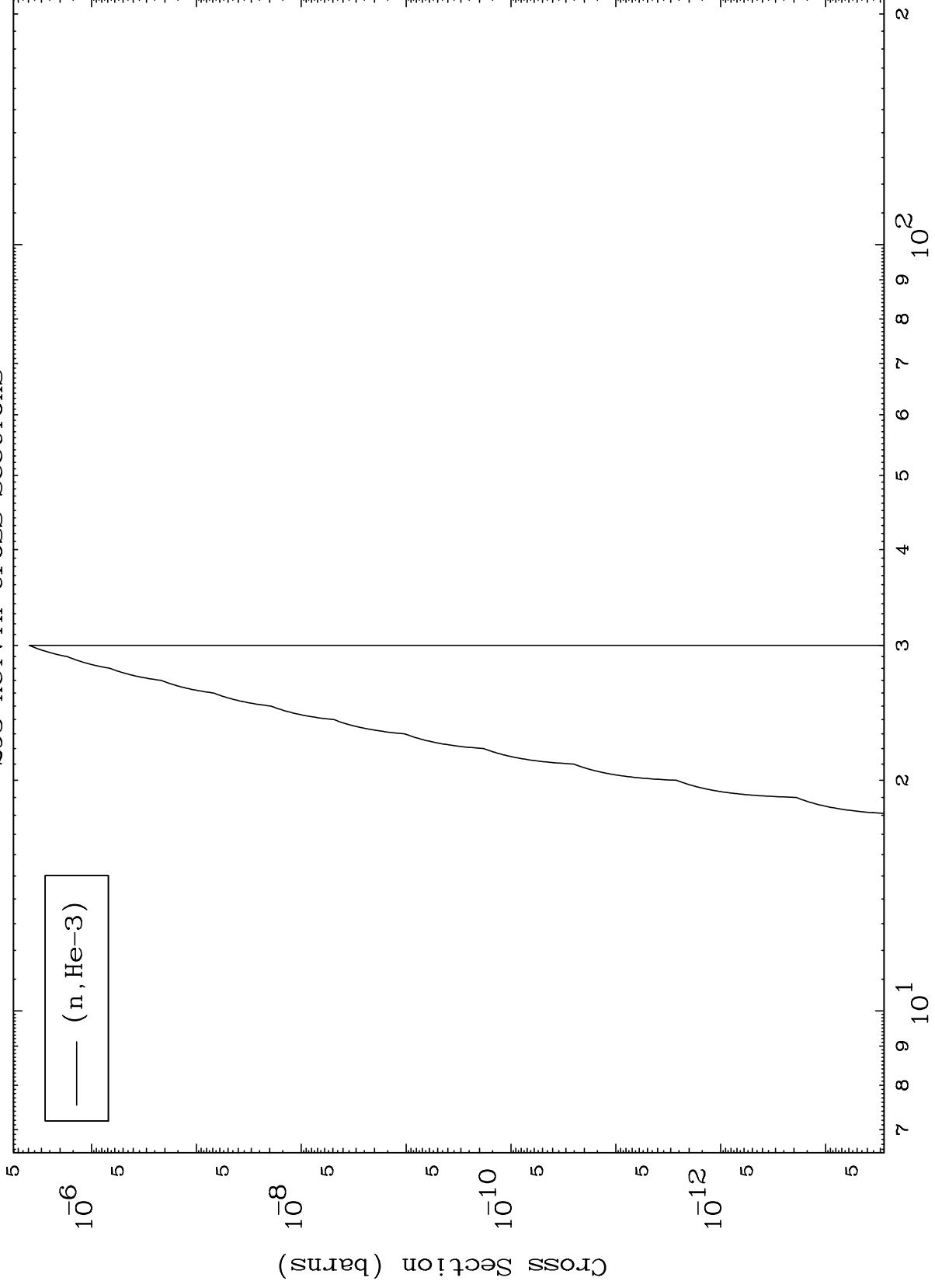
Incident Energy (MeV)

94-Pu-247

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(n,He3) Levels  
293 Kelvin Cross Sections

94-Pu-247



16

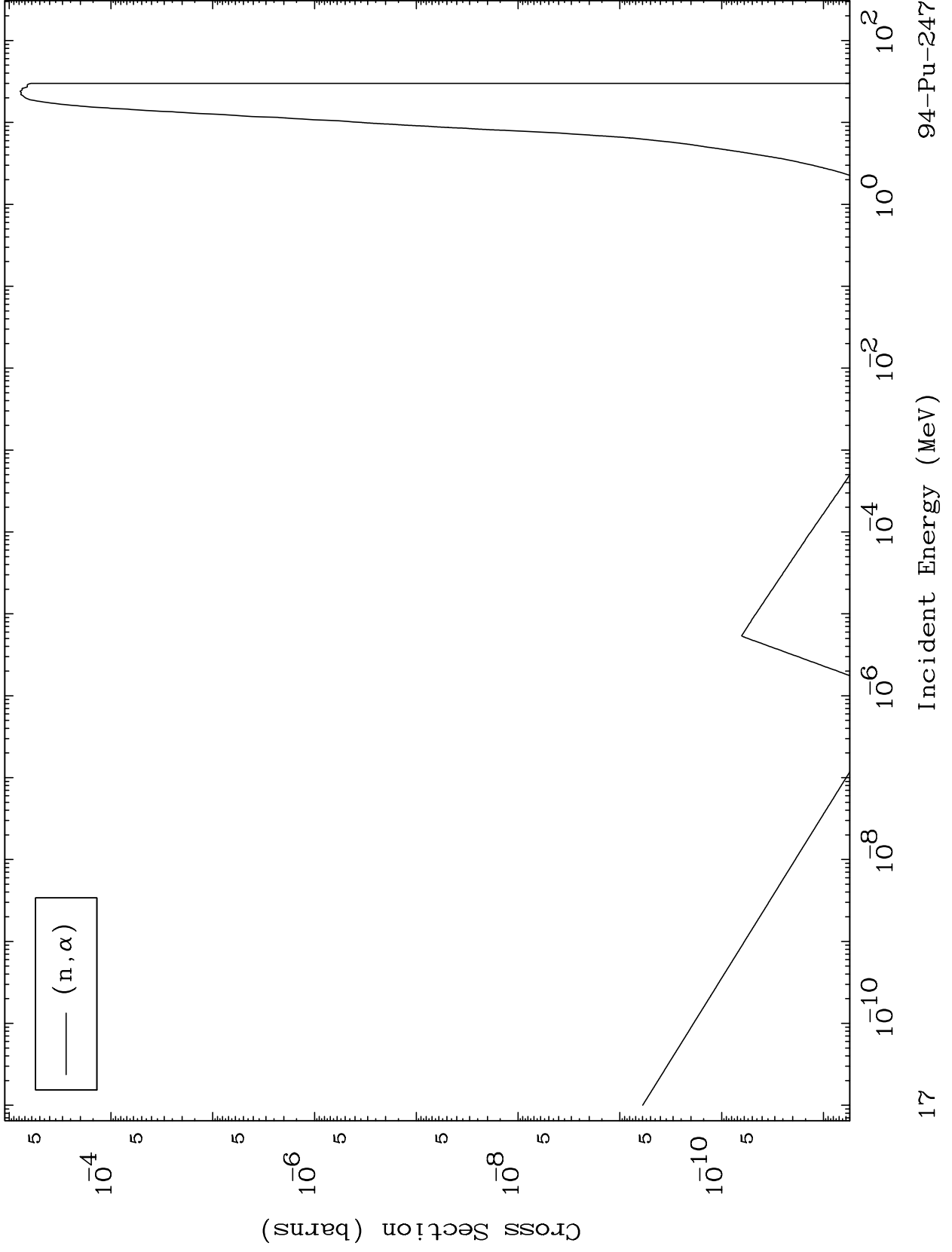
Incident Energy (MeV)

94-Pu-247

MAT 9461

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

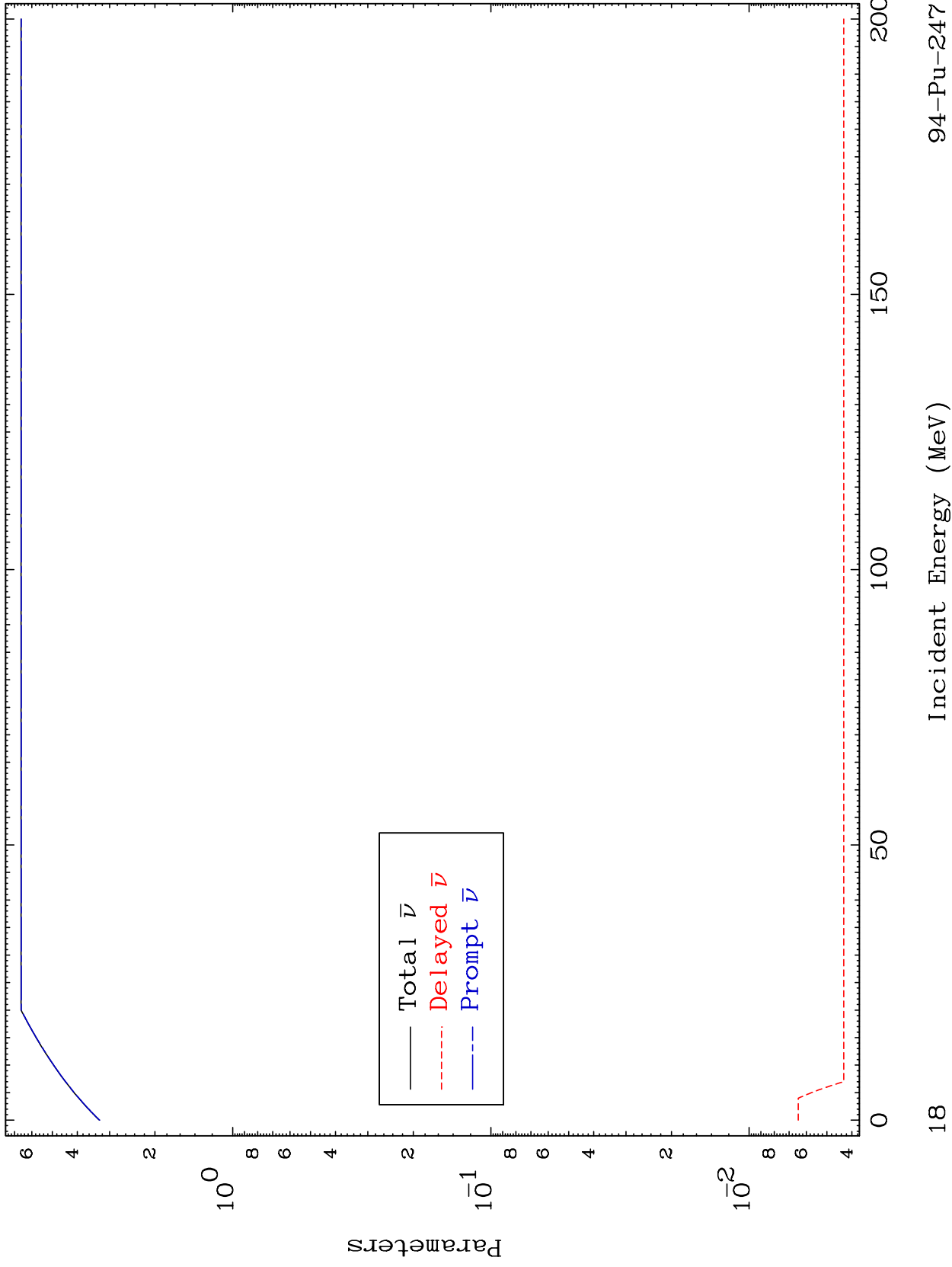
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Energy Release  
Parameters

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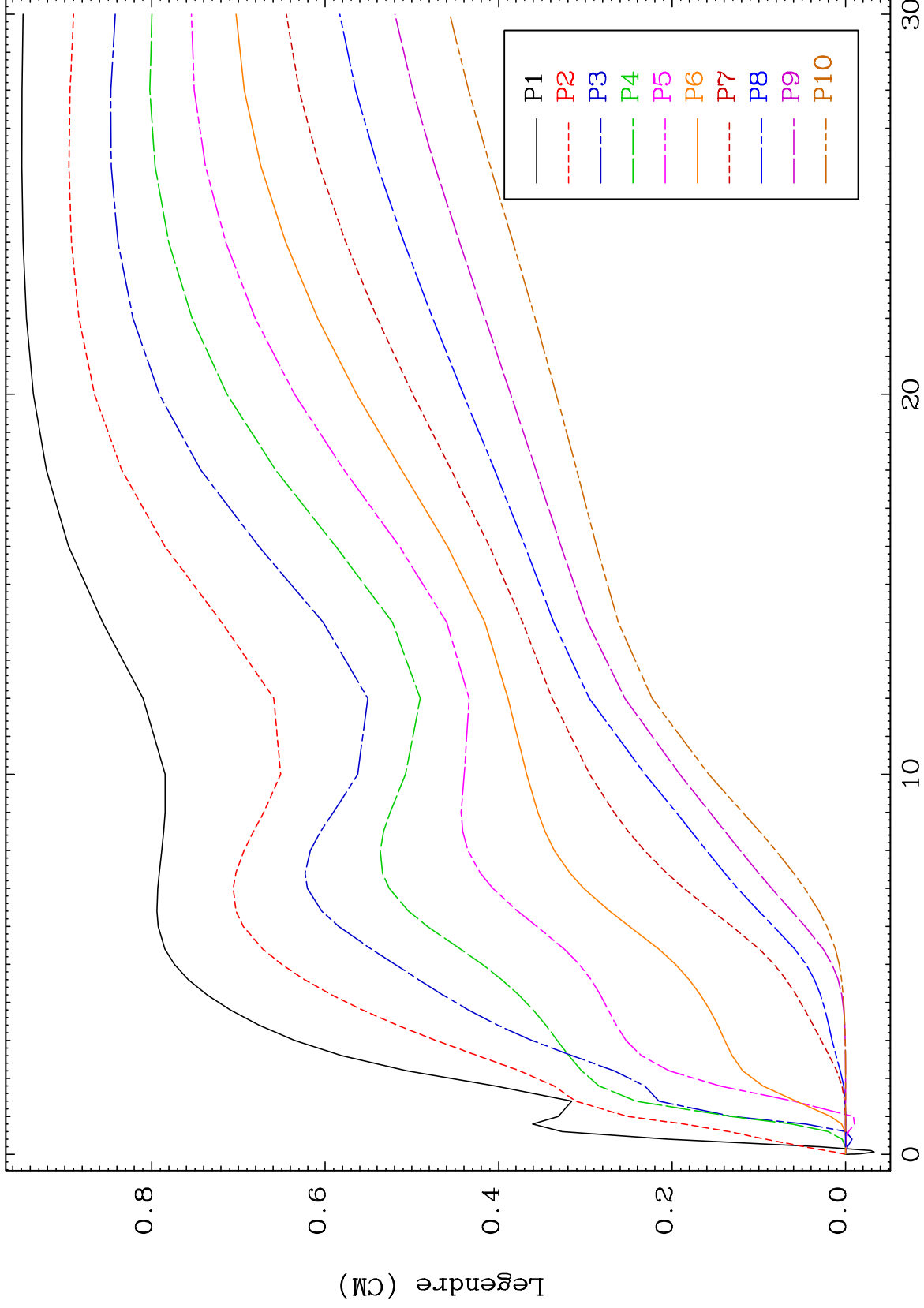
Incident Energy (MeV)

94-Pu-247

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Elastic Legendre Coefficients

94-Pu-247



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Incident Energy (MeV)

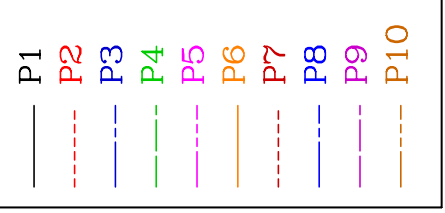
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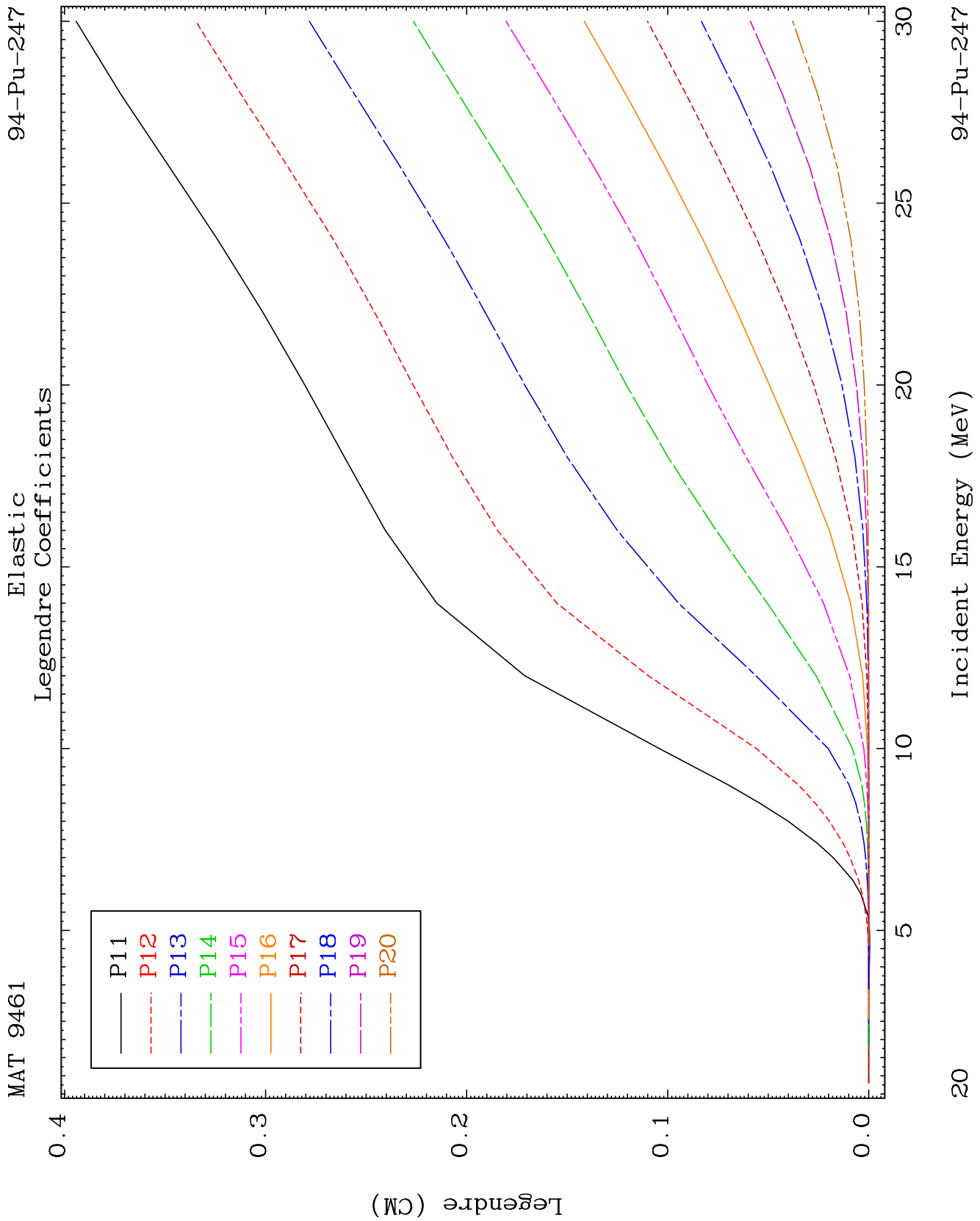
30

20

10

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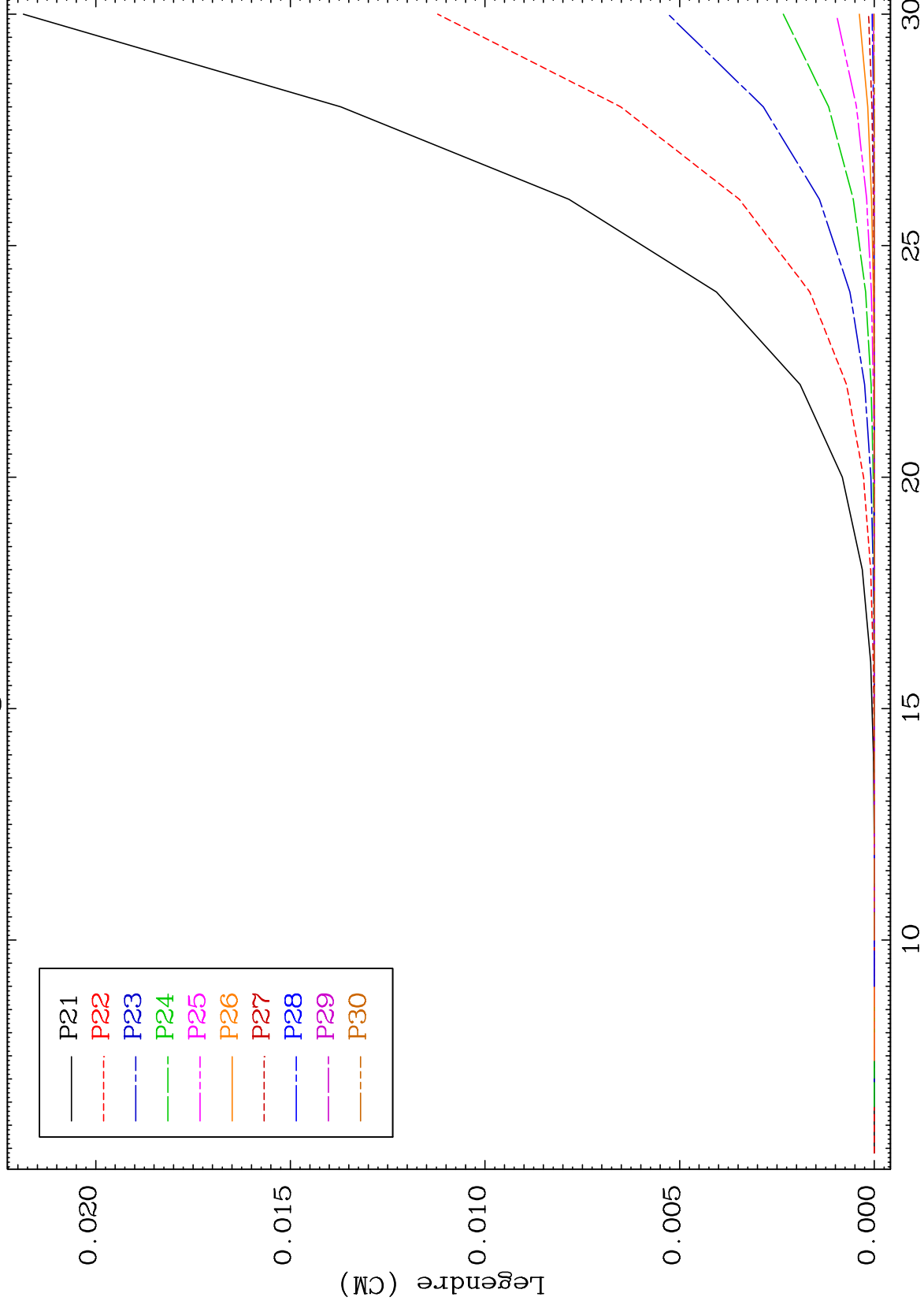




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### Elastic Legendre Coefficients

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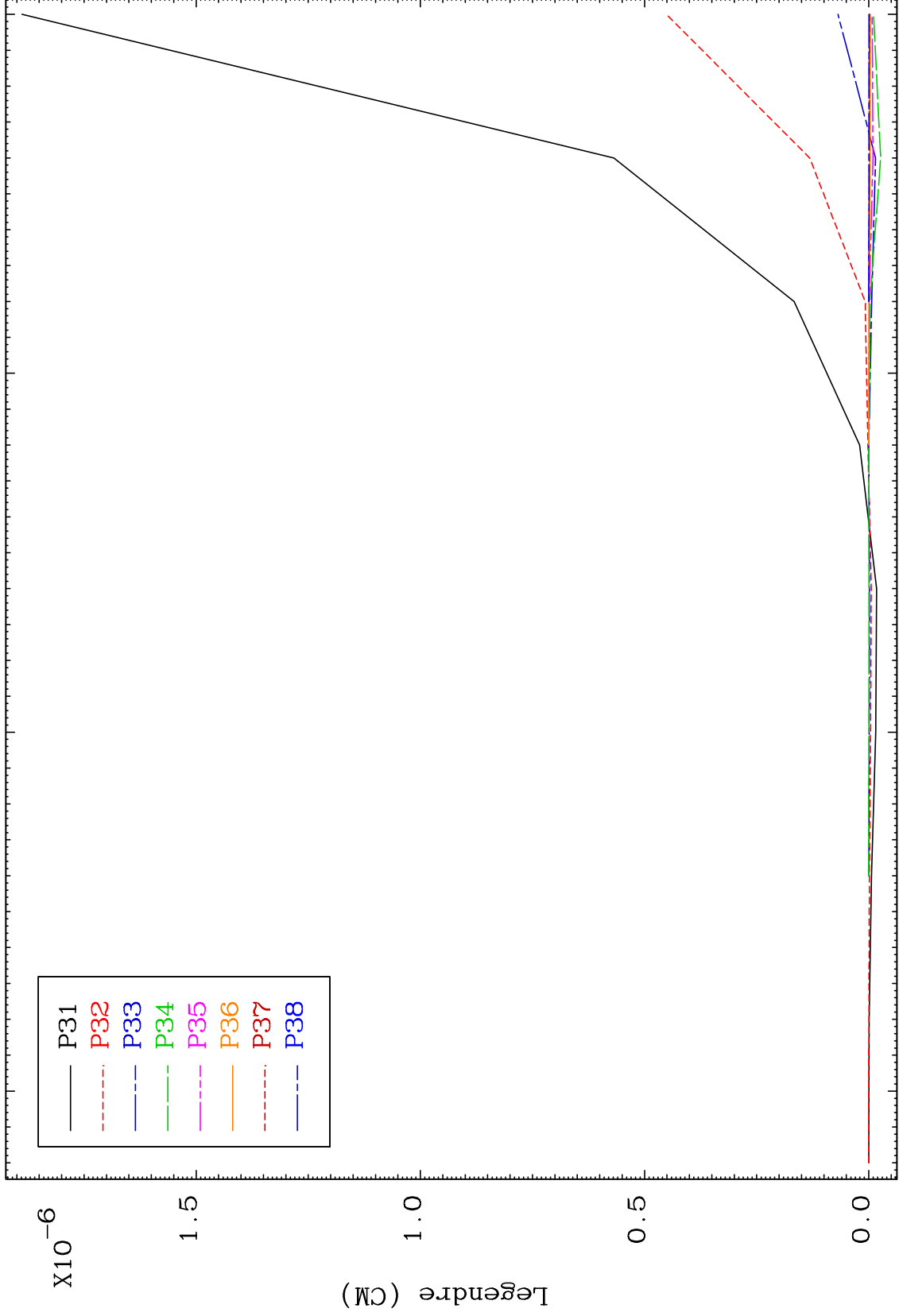
Incident Energy (MeV)

94-Pu-247

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Elastic Legendre Coefficients

94-Pu-247



22

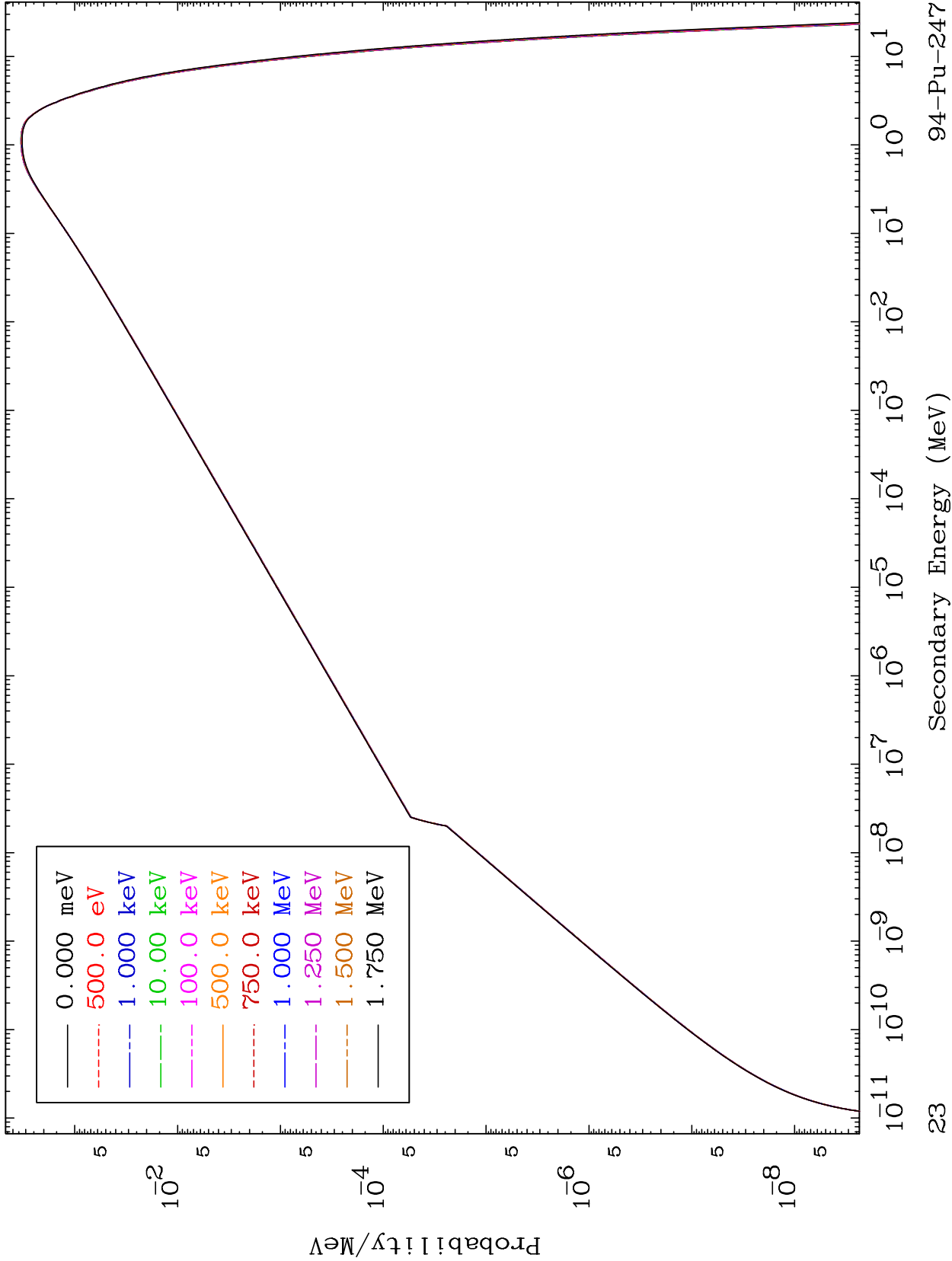
Incident Energy (MeV)

94-Pu-247

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### Fission Energy Distributions

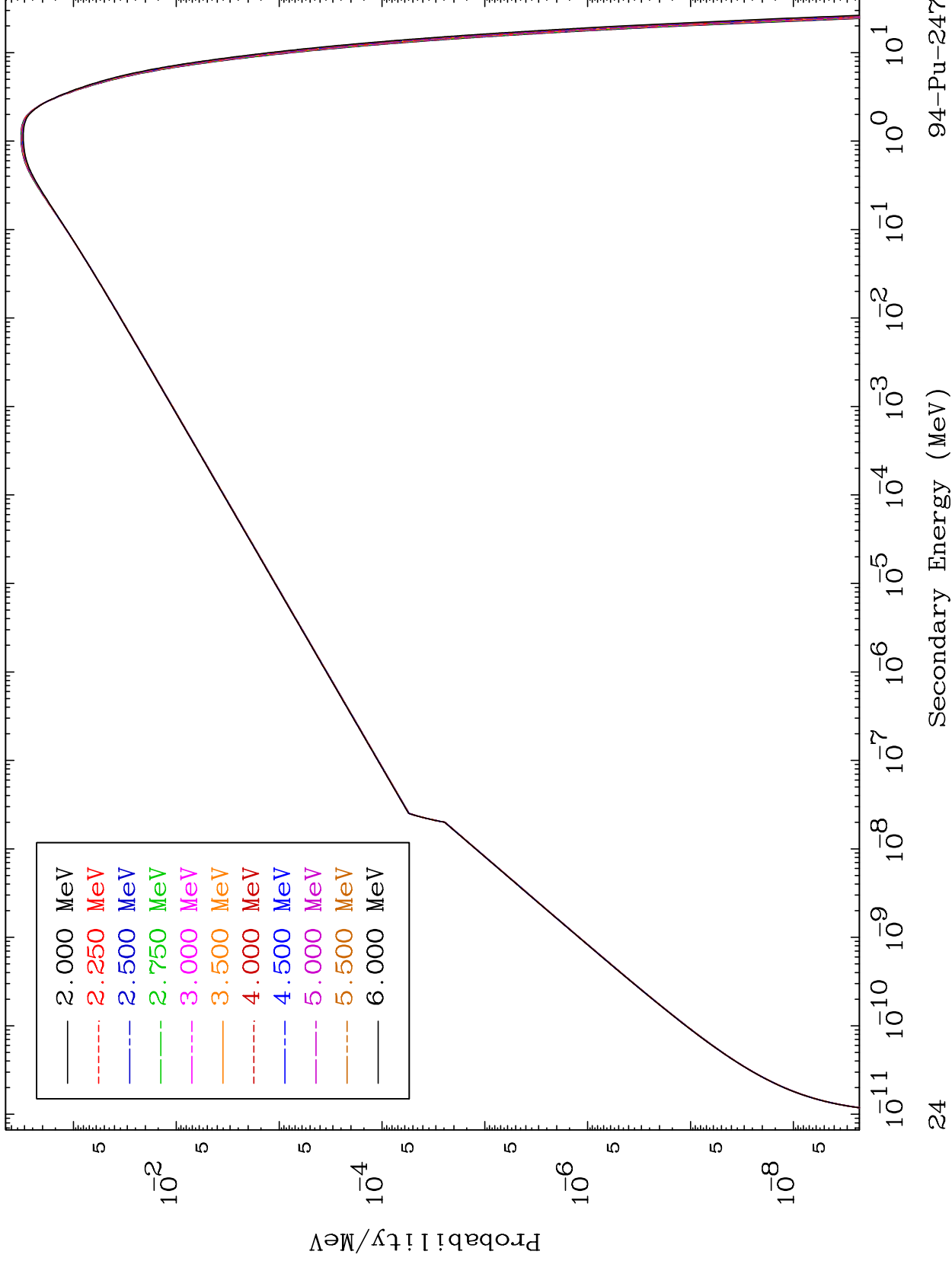
94-Pu-247



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### Fission Energy Distributions

94-Pu-247



94-Pu-247

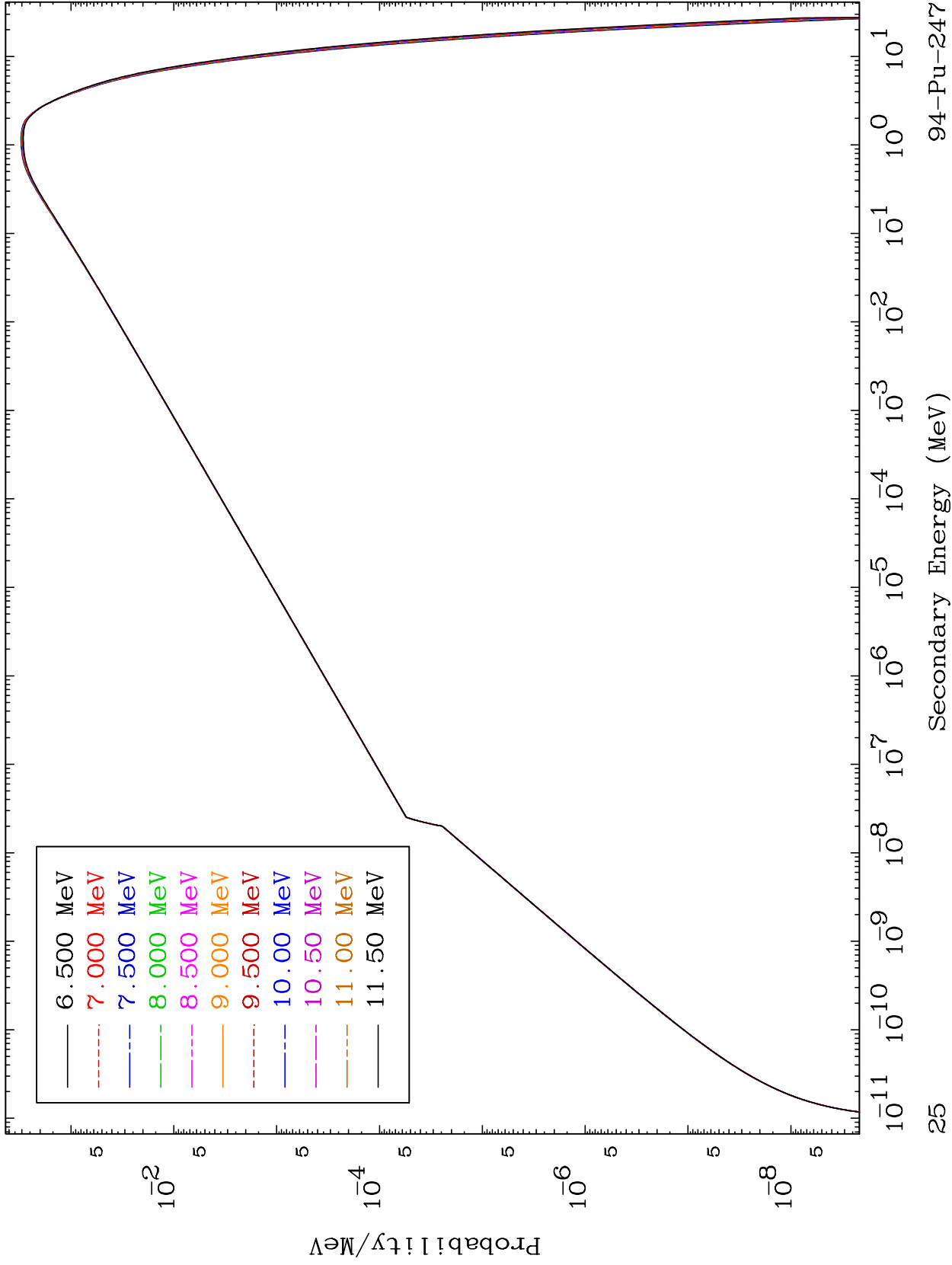
Secondary Energy (MeV)

24

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### Fission Energy Distributions

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25

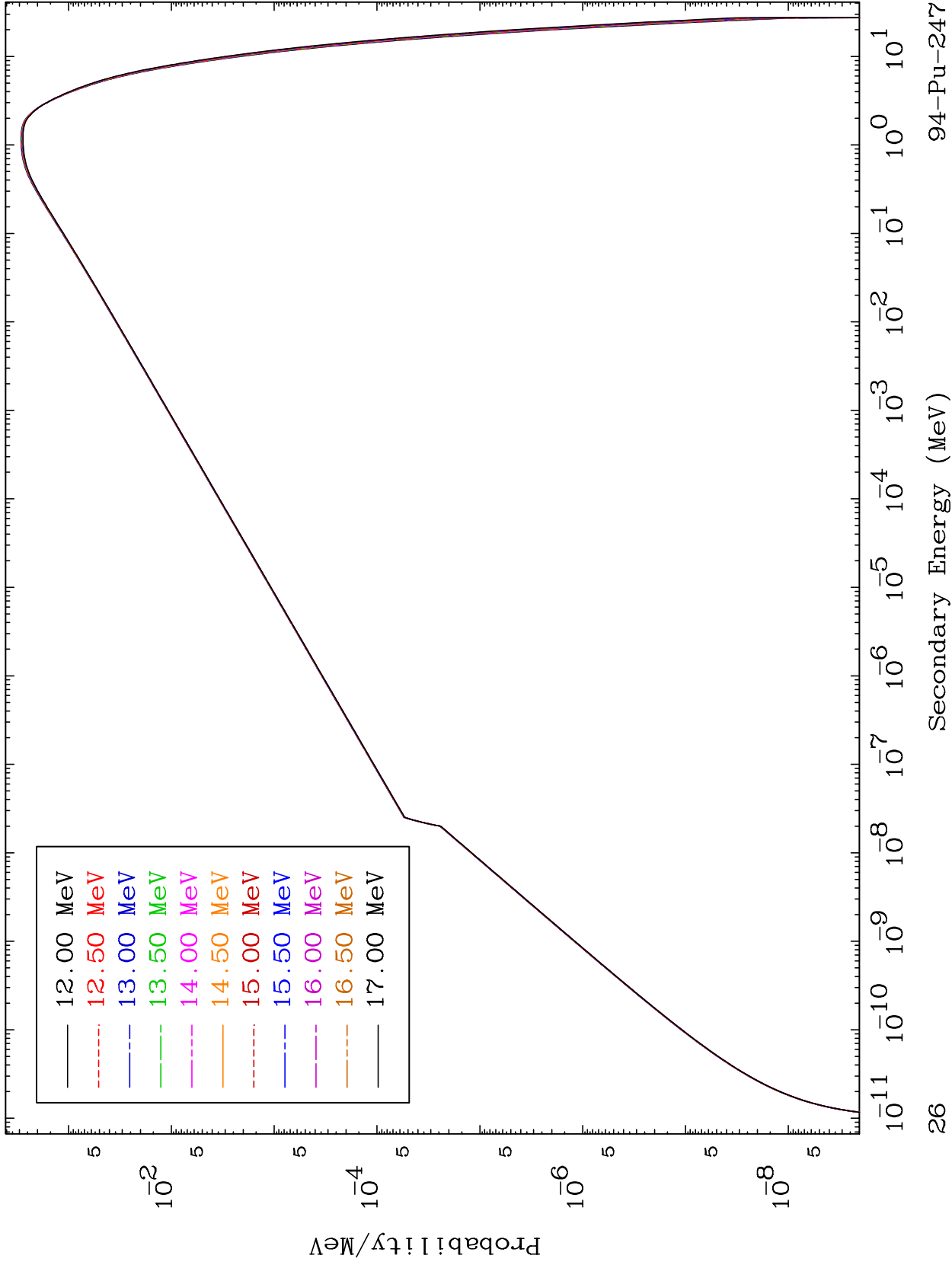
Secondary Energy (MeV)

94-Pu-247

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### Fission Energy Distributions

94-Pu-247



94-Pu-247

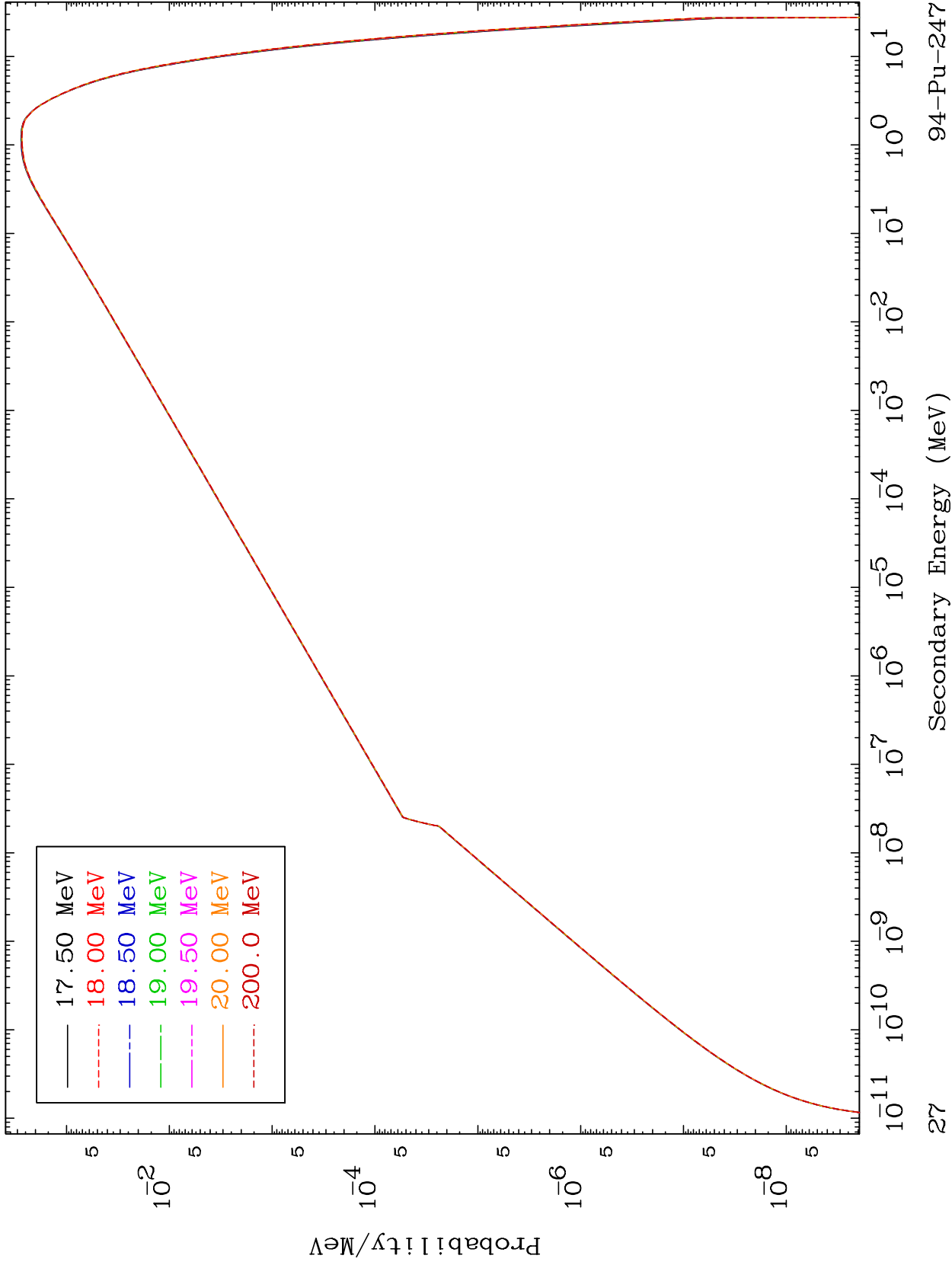
Secondary Energy (MeV)

26

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### Fission Energy Distributions

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Secondary Energy (MeV)

27

(n,4n)  
Radionuclide Production Cross Section

