

Program EVALPLOT
(Version 2021-1)

by

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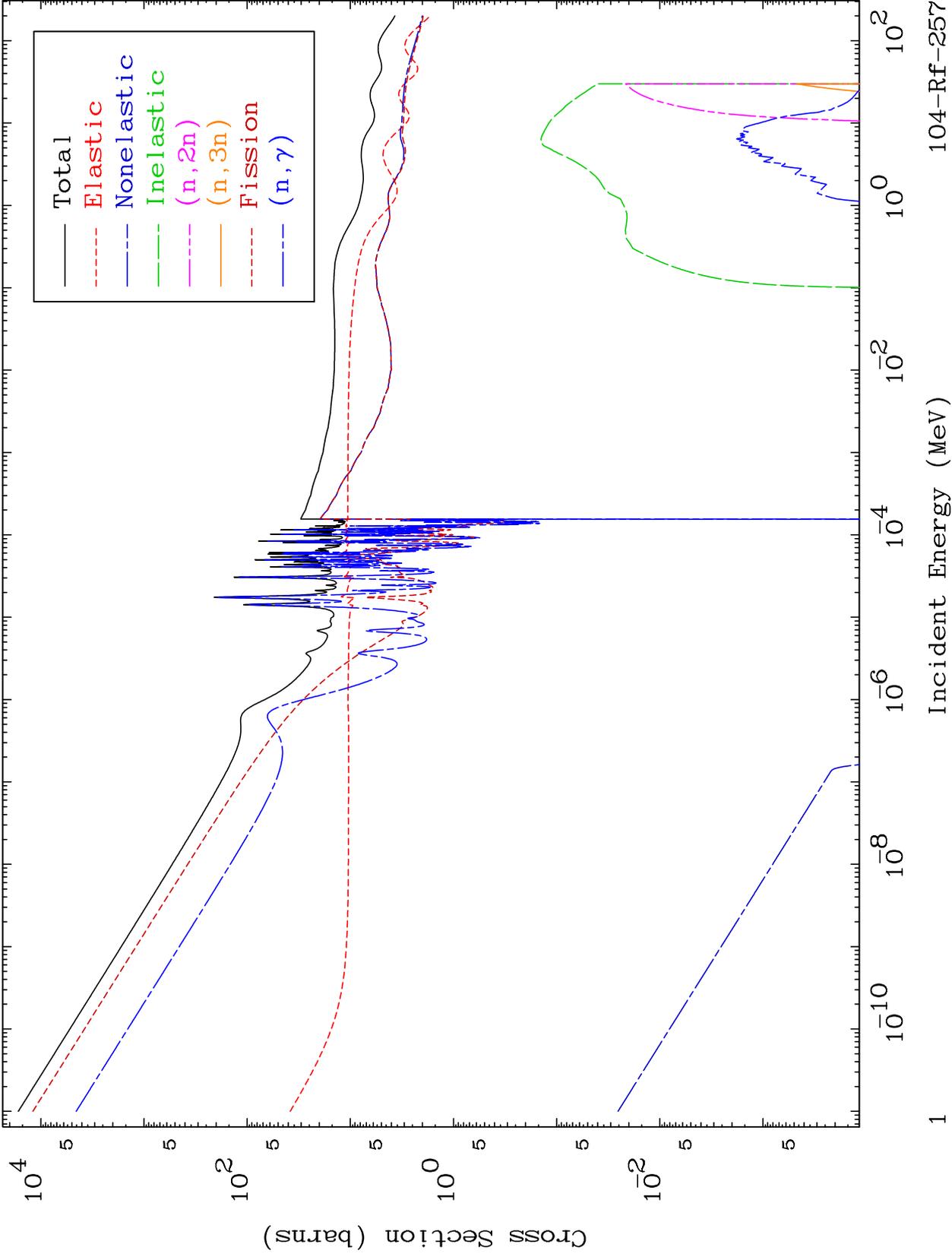
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Press Mouse Button to Start

MAT 457

Neutron Major
293 Kelvin Cross Sections

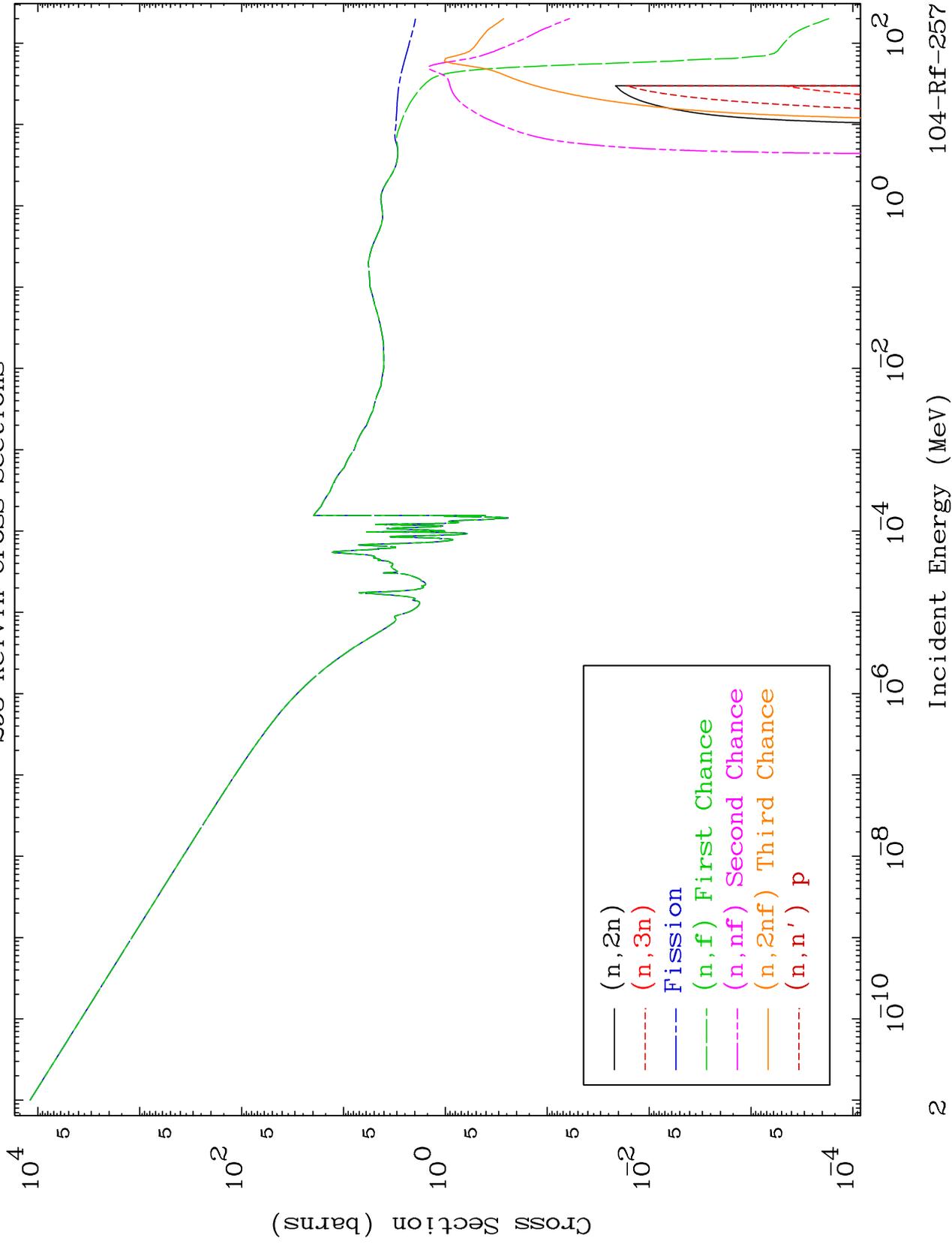
104-Rf-257

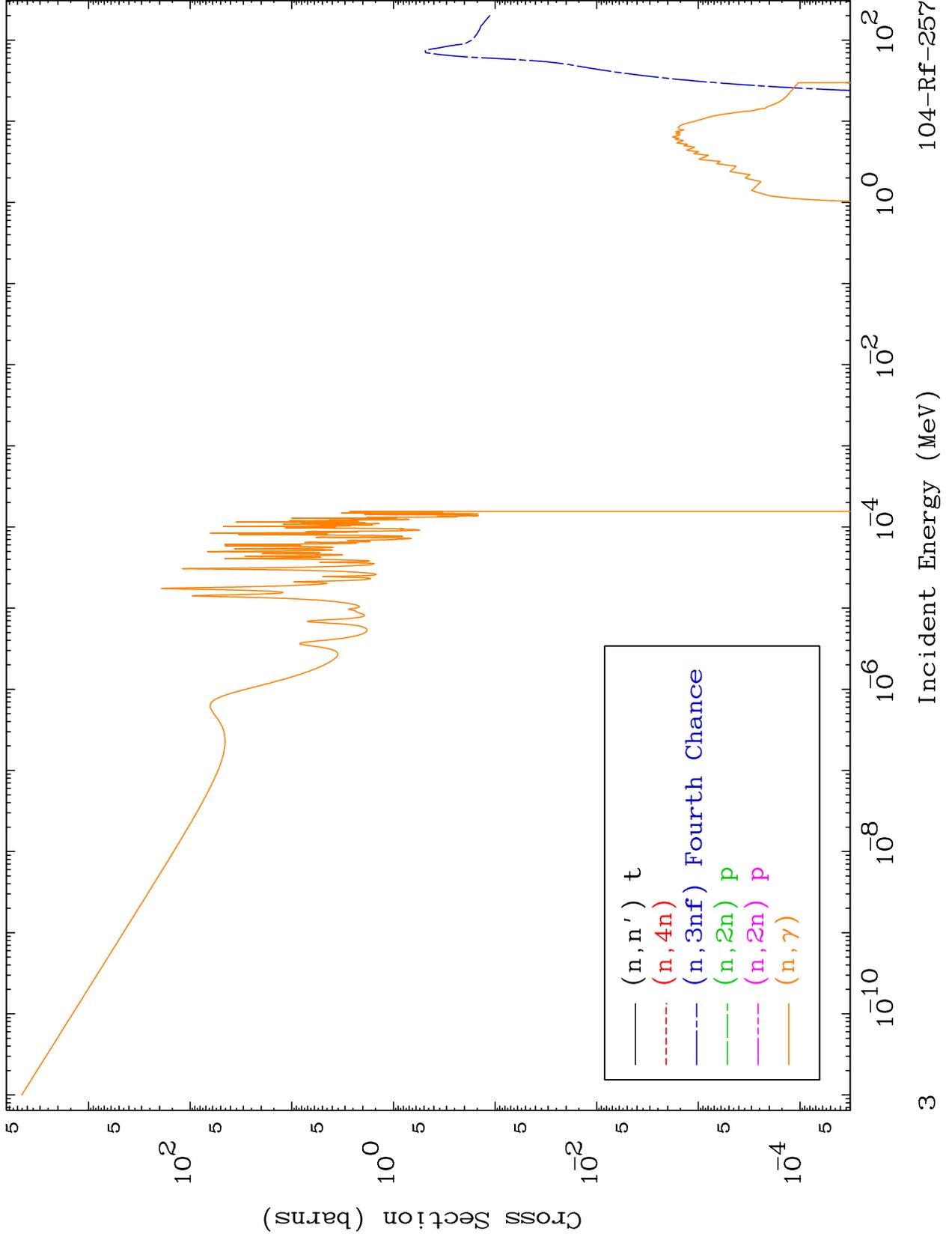


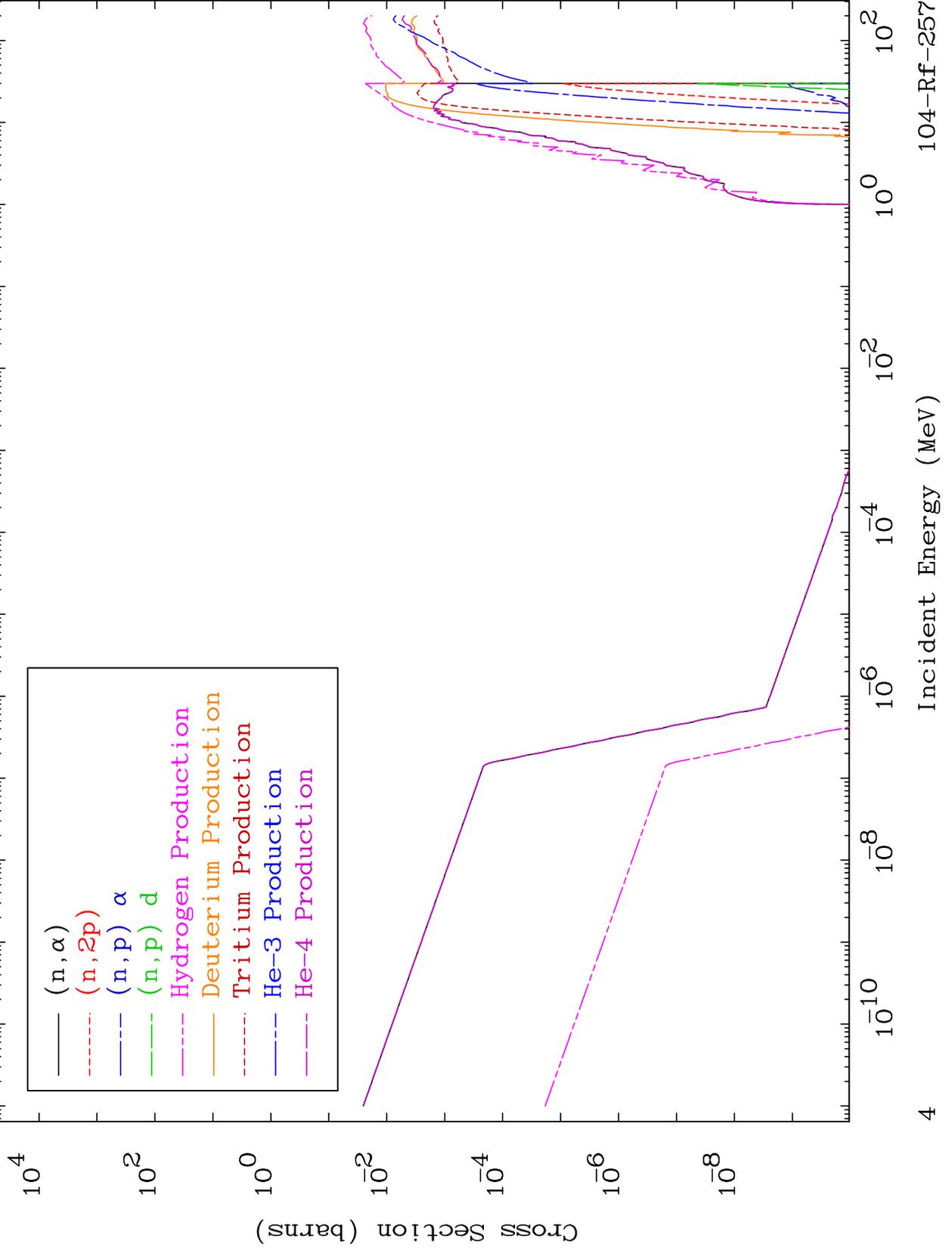
MAT 457

Neutron Absorption
293 Kelvin Cross Sections

104-Rf-257



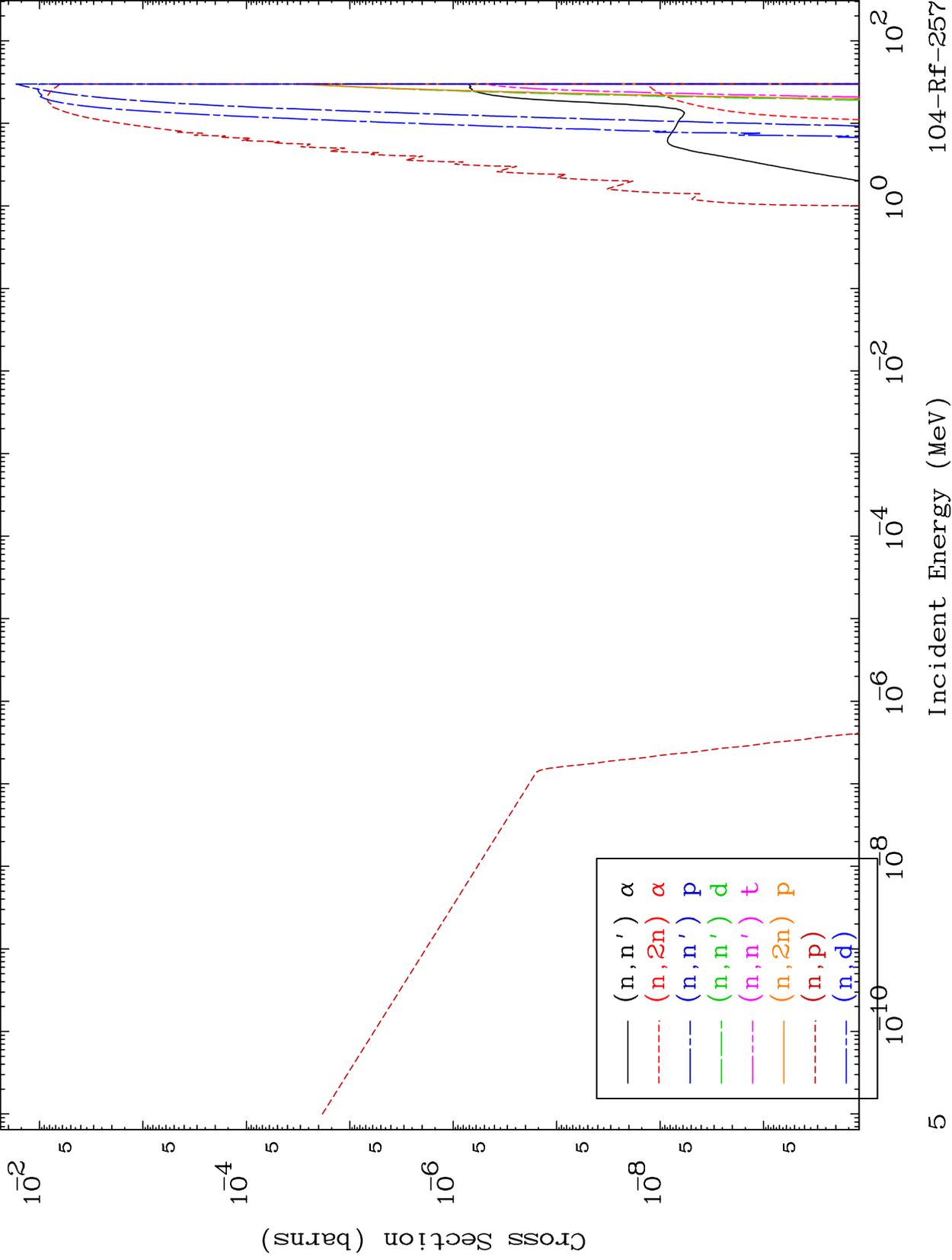




MAT 457

Charged Particle
293 Kelvin Cross Sections

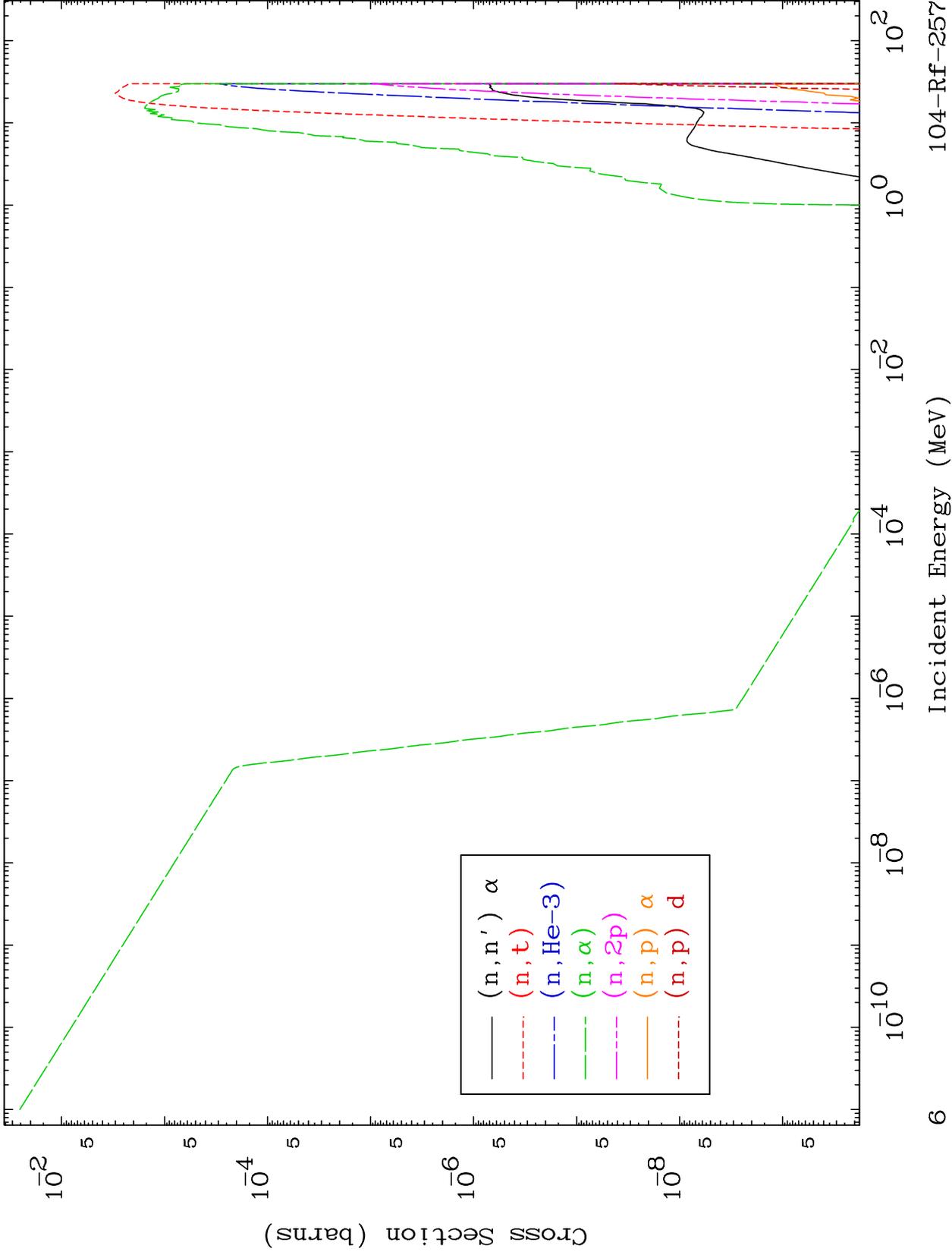
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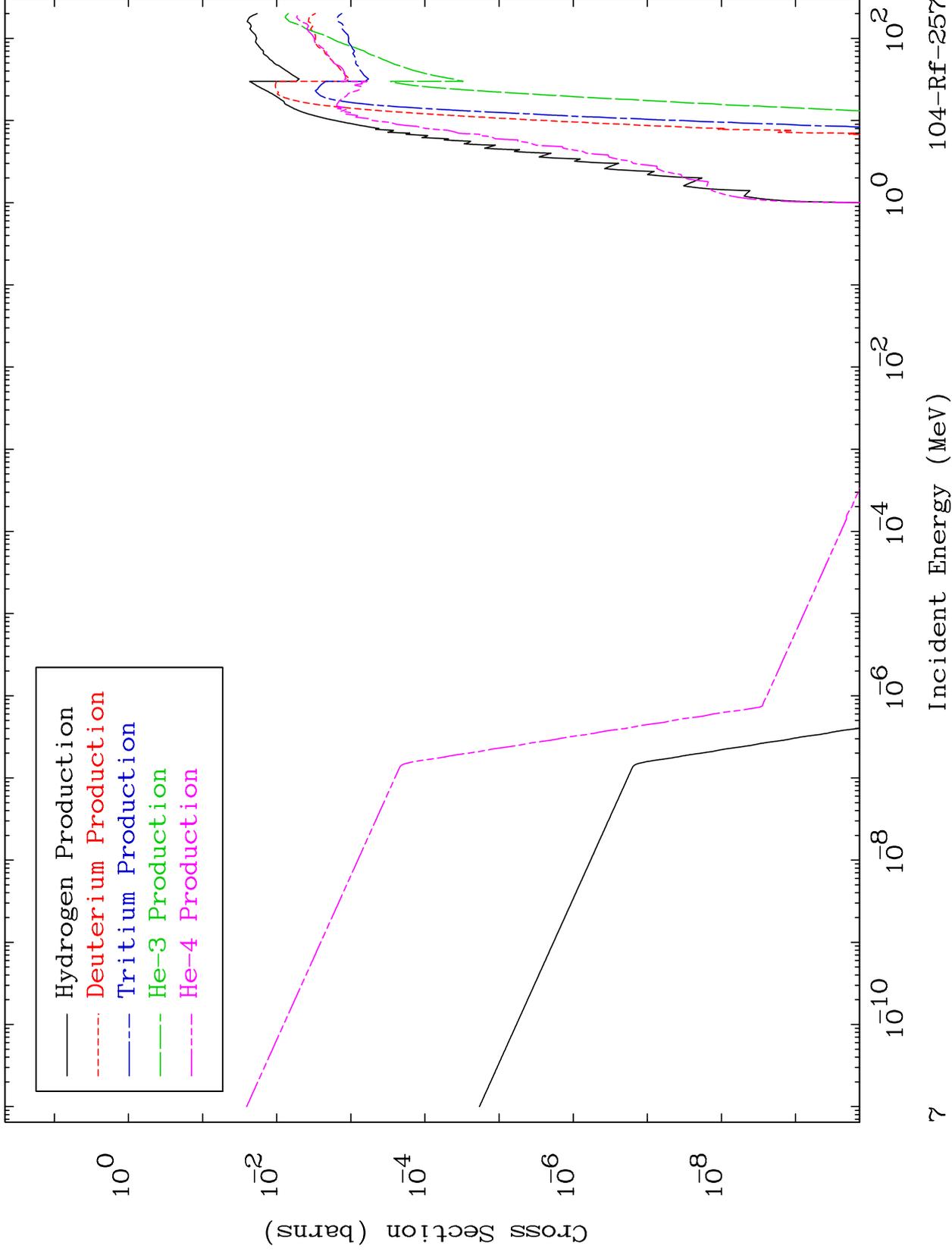


MAT 457

Charged Particle
293 Kelvin Cross Sections

104-Rf-257

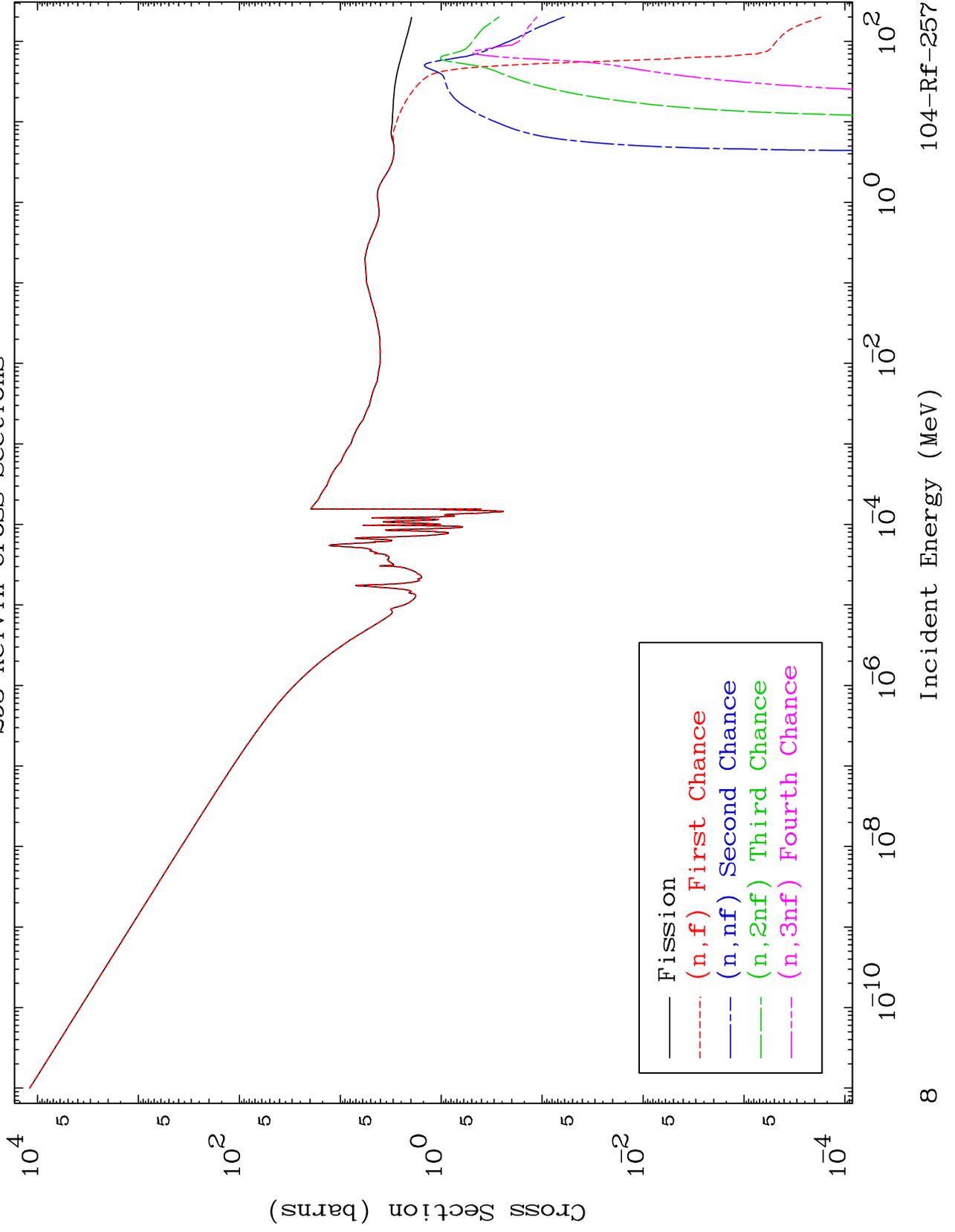




MAT 457

Fission
293 Kelvin Cross Sections

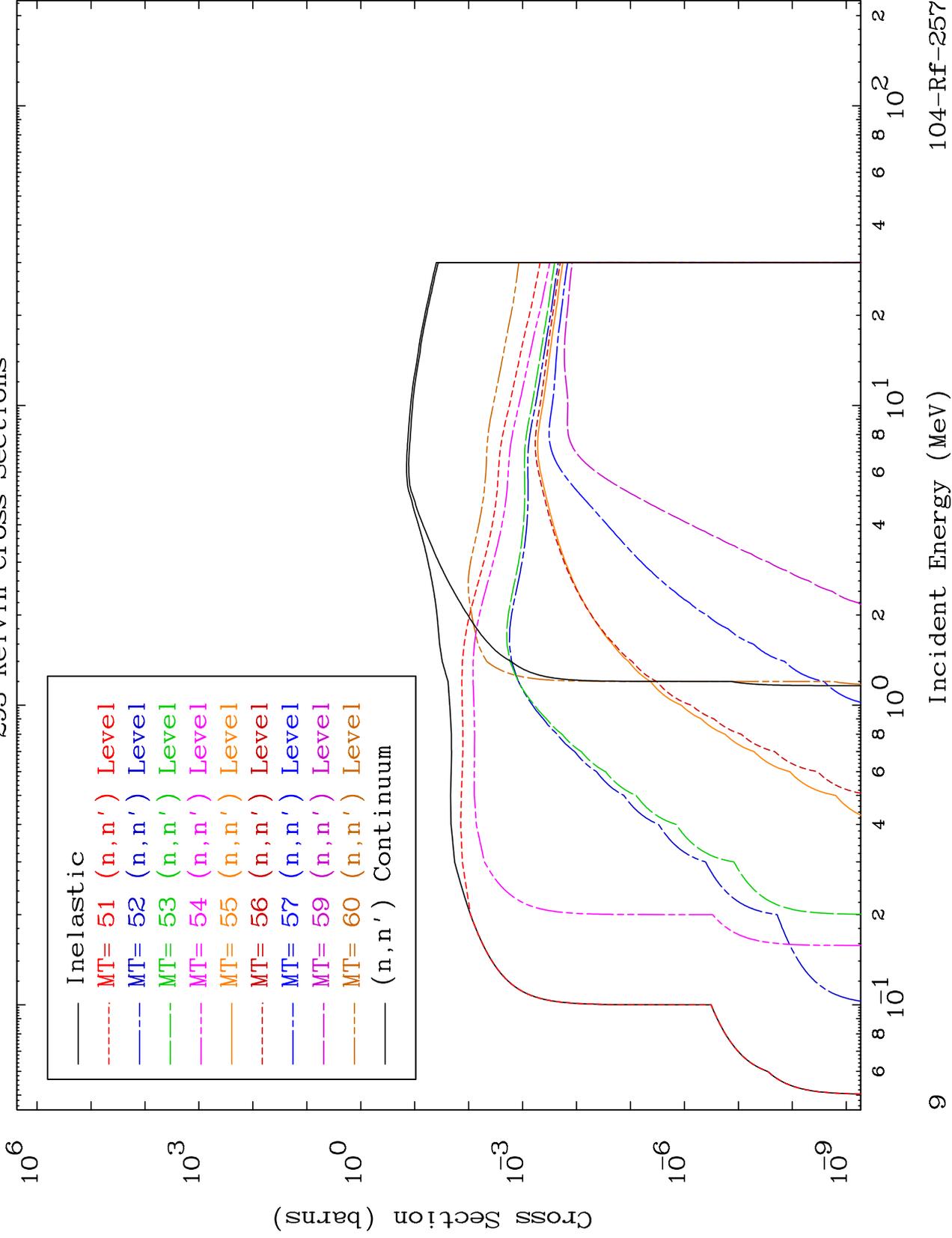
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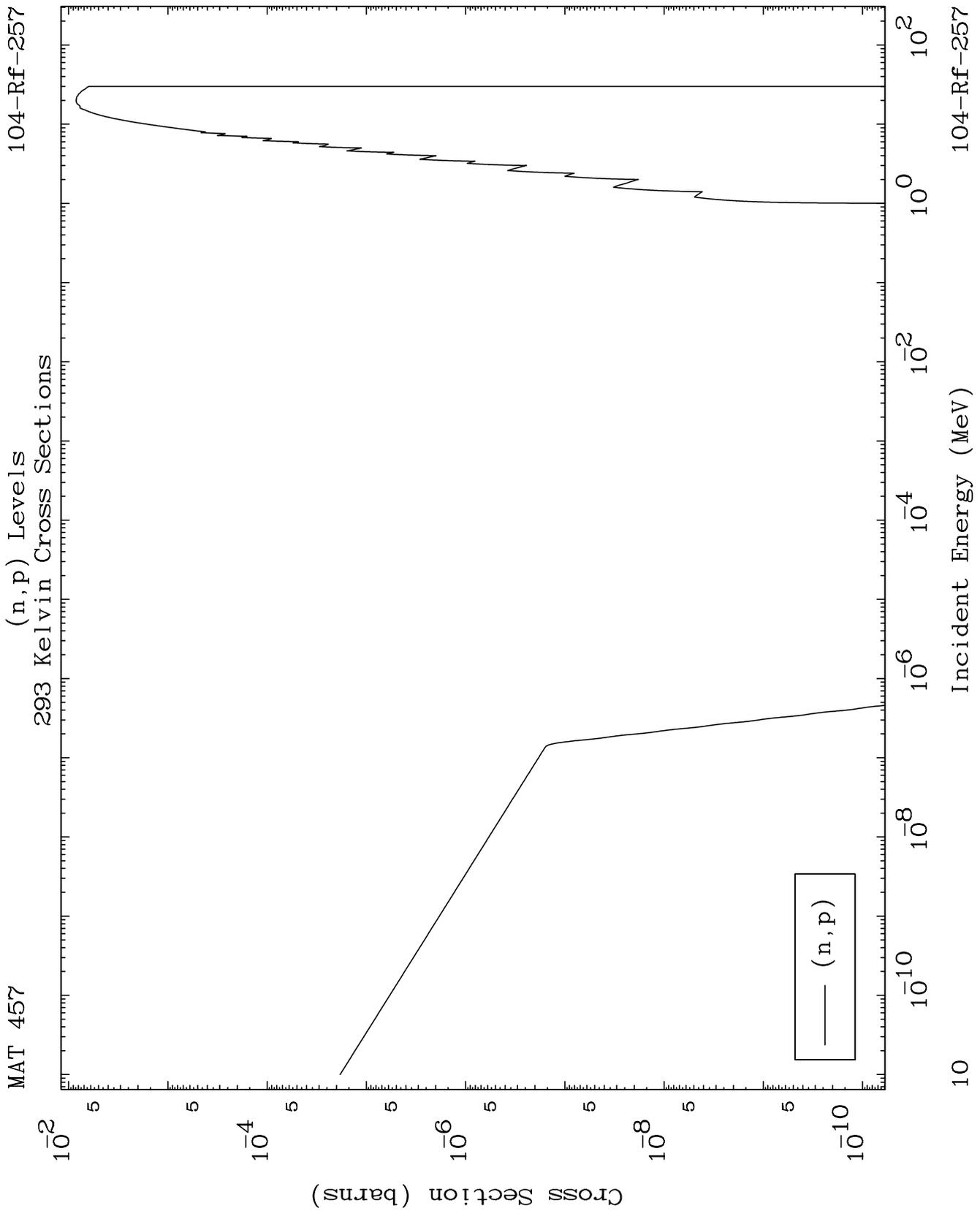


MAT 457

(n,n') Levels
293 Kelvin Cross Sections

104-Rf-257

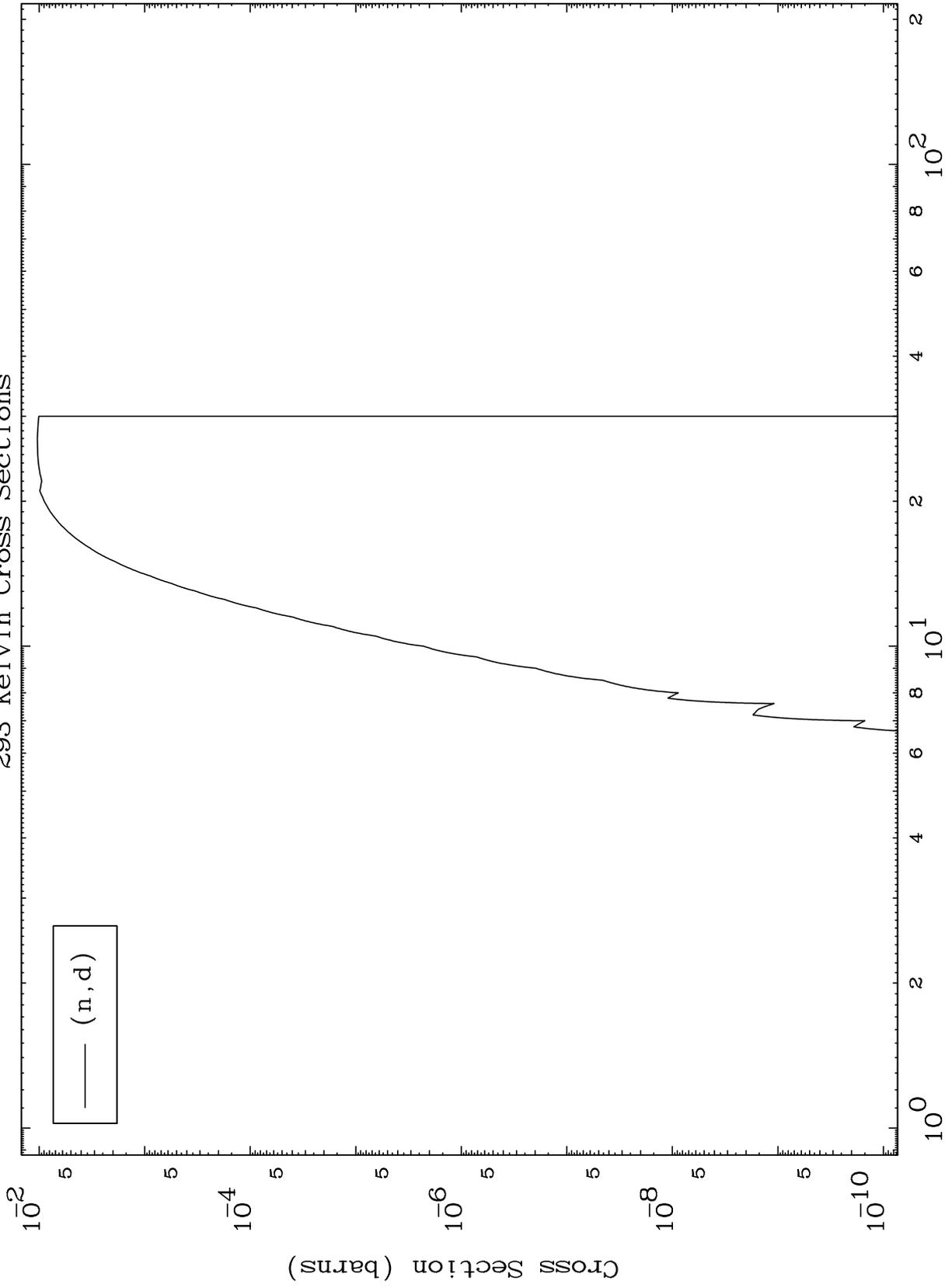




MAT 457

(n,d) Levels
293 Kelvin Cross Sections

104-Rf-257



11

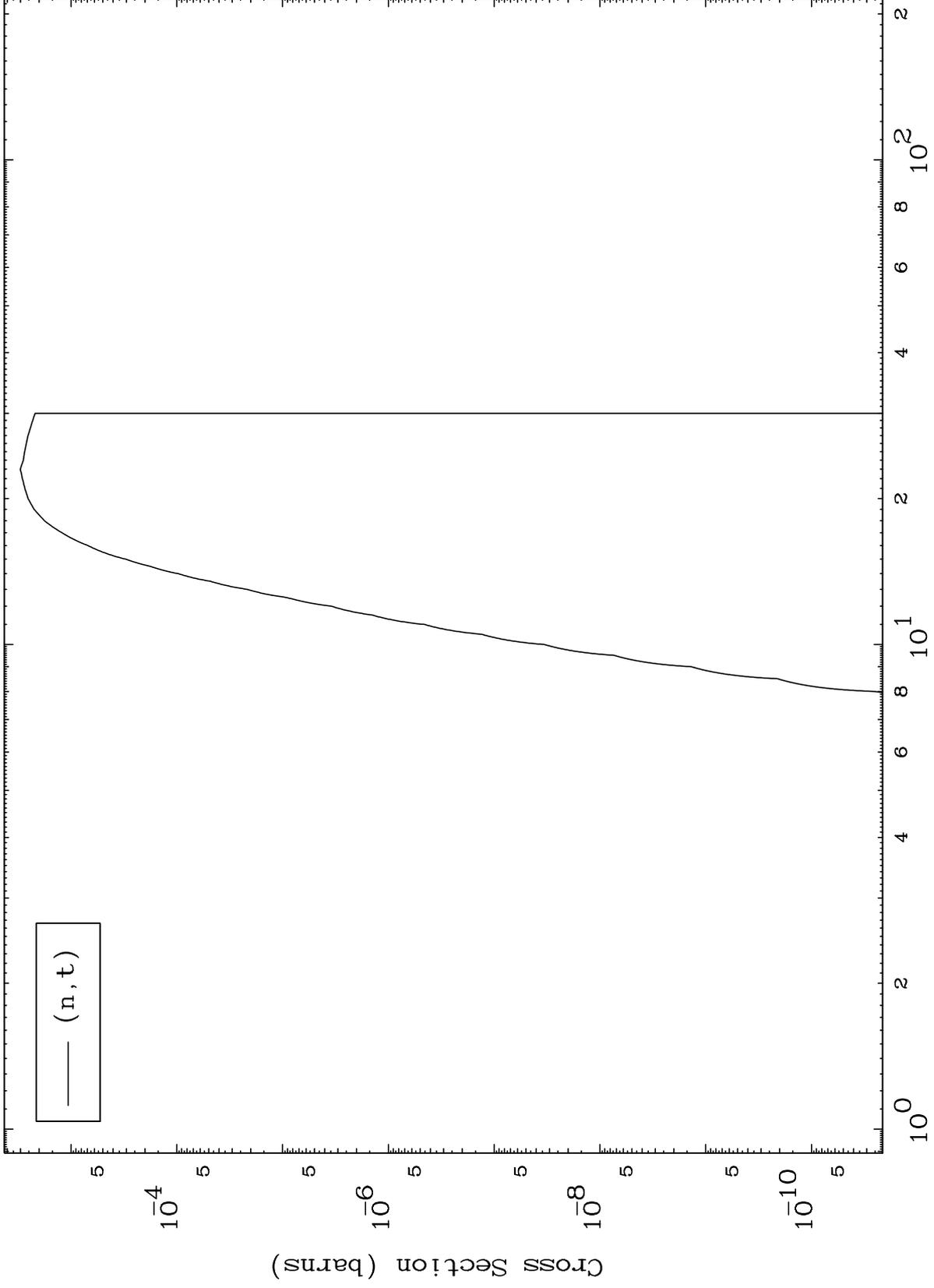
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,t) Levels
293 Kelvin Cross Sections

104-Rf-257



Incident Energy (MeV)

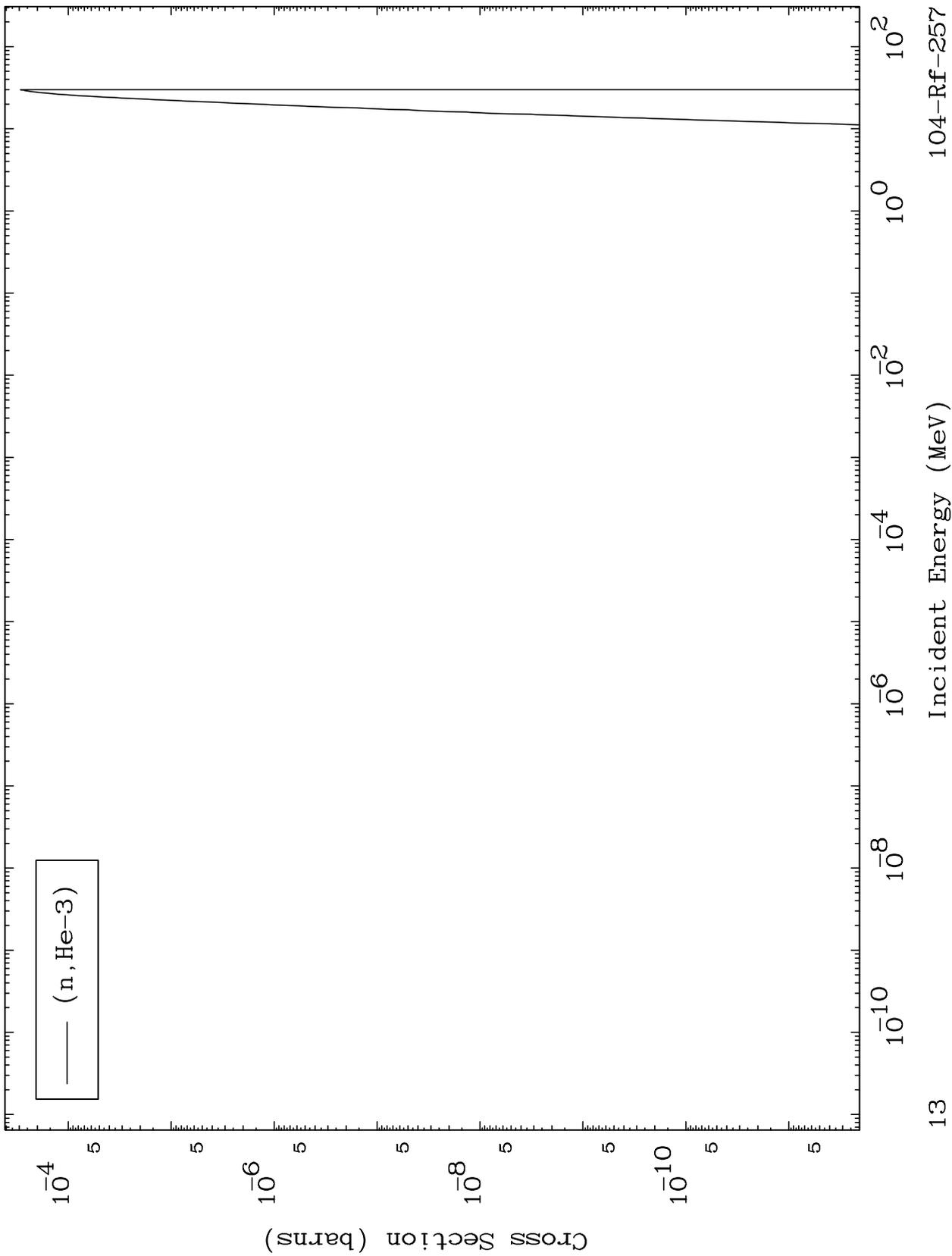
104-Rf-257

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MAT 457

(n,He3) Levels
293 Kelvin Cross Sections

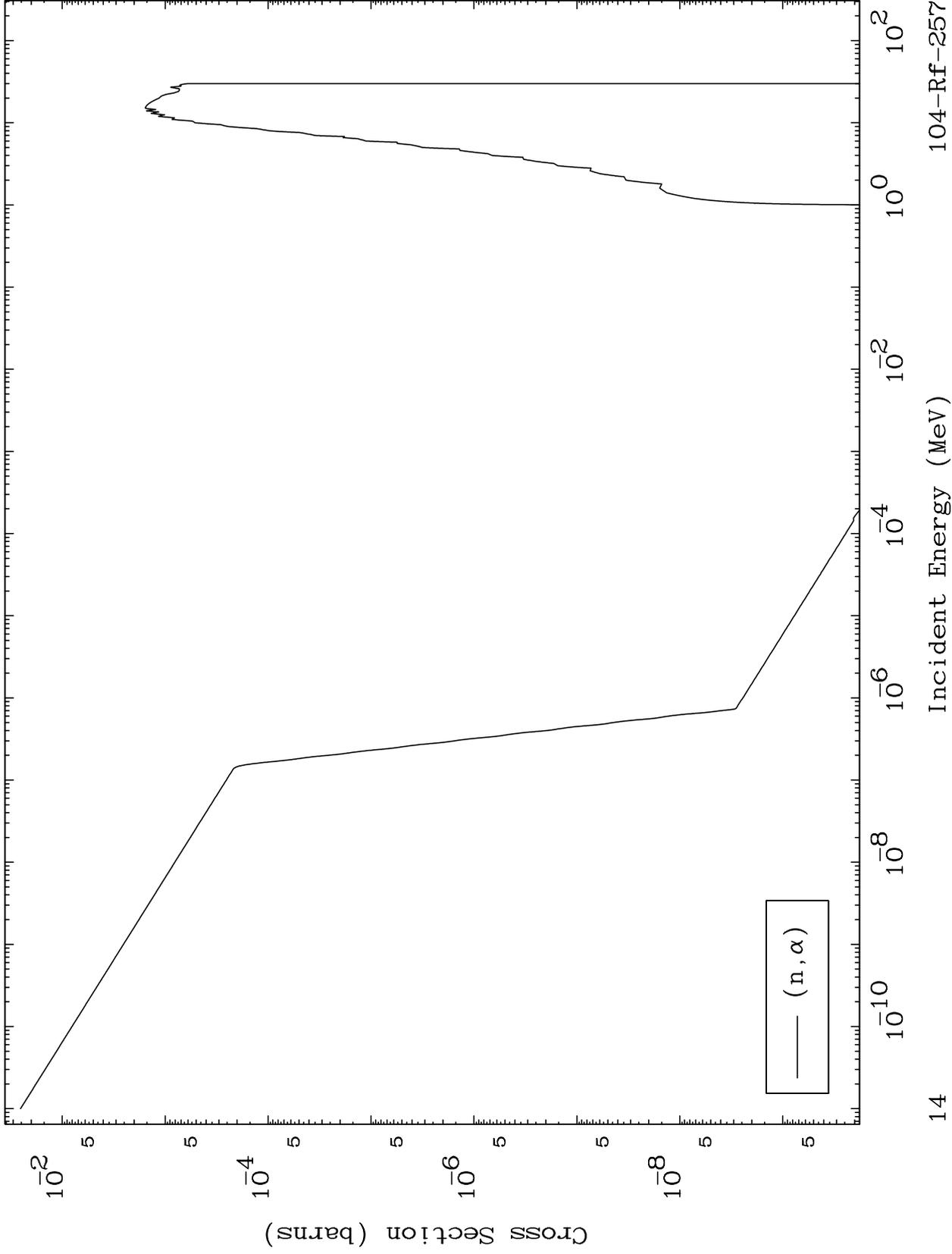
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MAT 457

(n, α) Levels
293 Kelvin Cross Sections

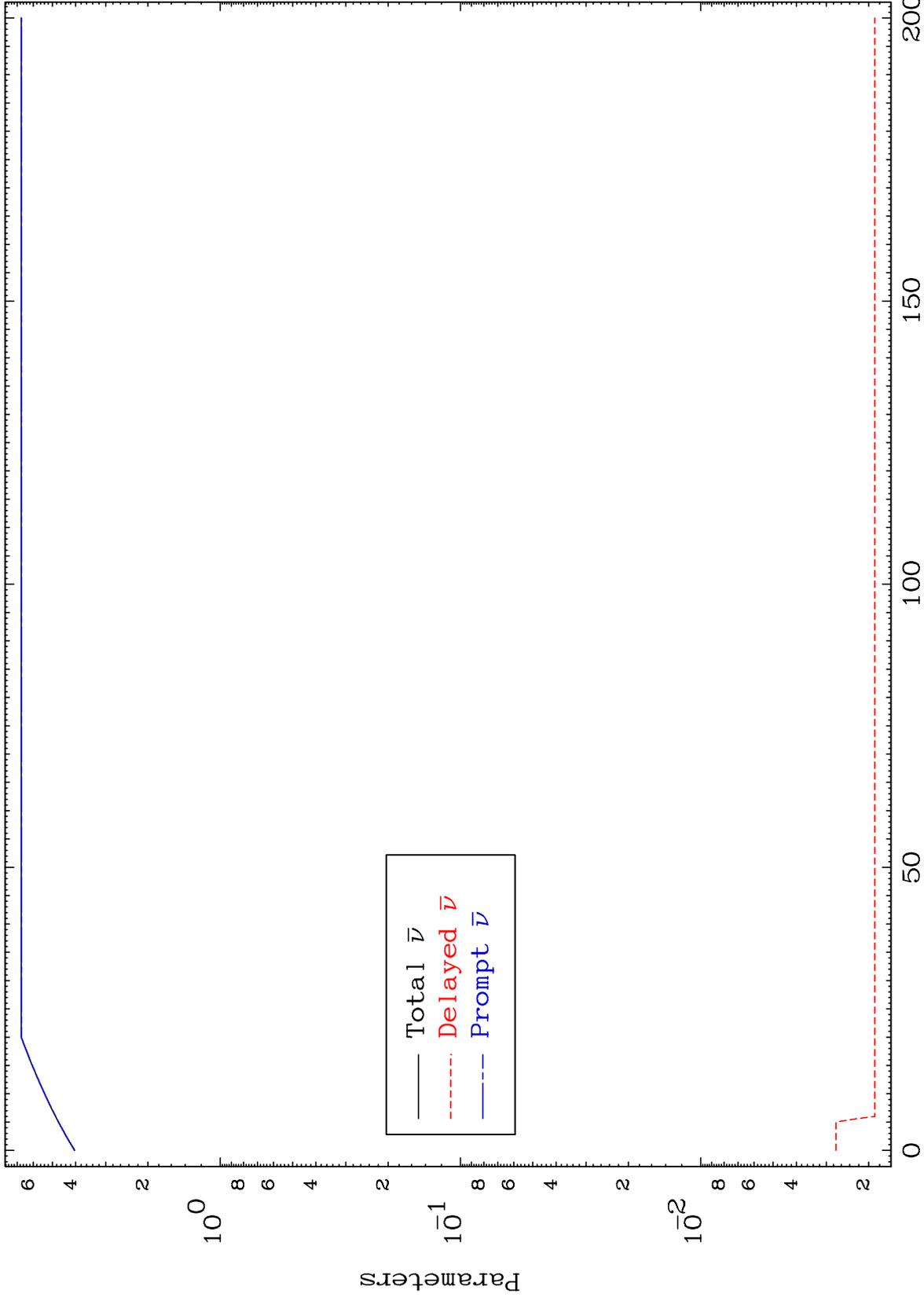
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MAT 457

Energy Release
Parameters

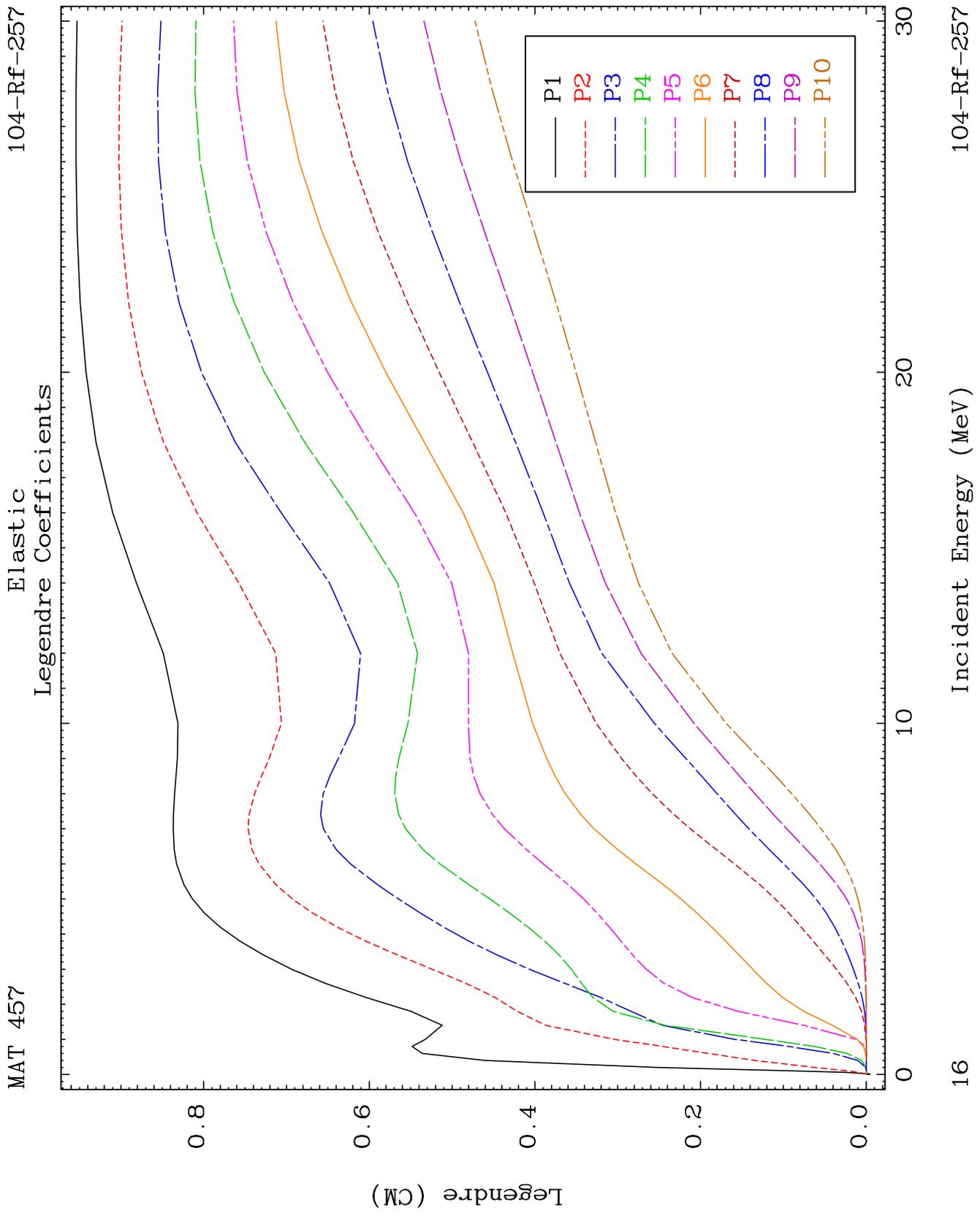
104-Rf-257



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Incident Energy (MeV)

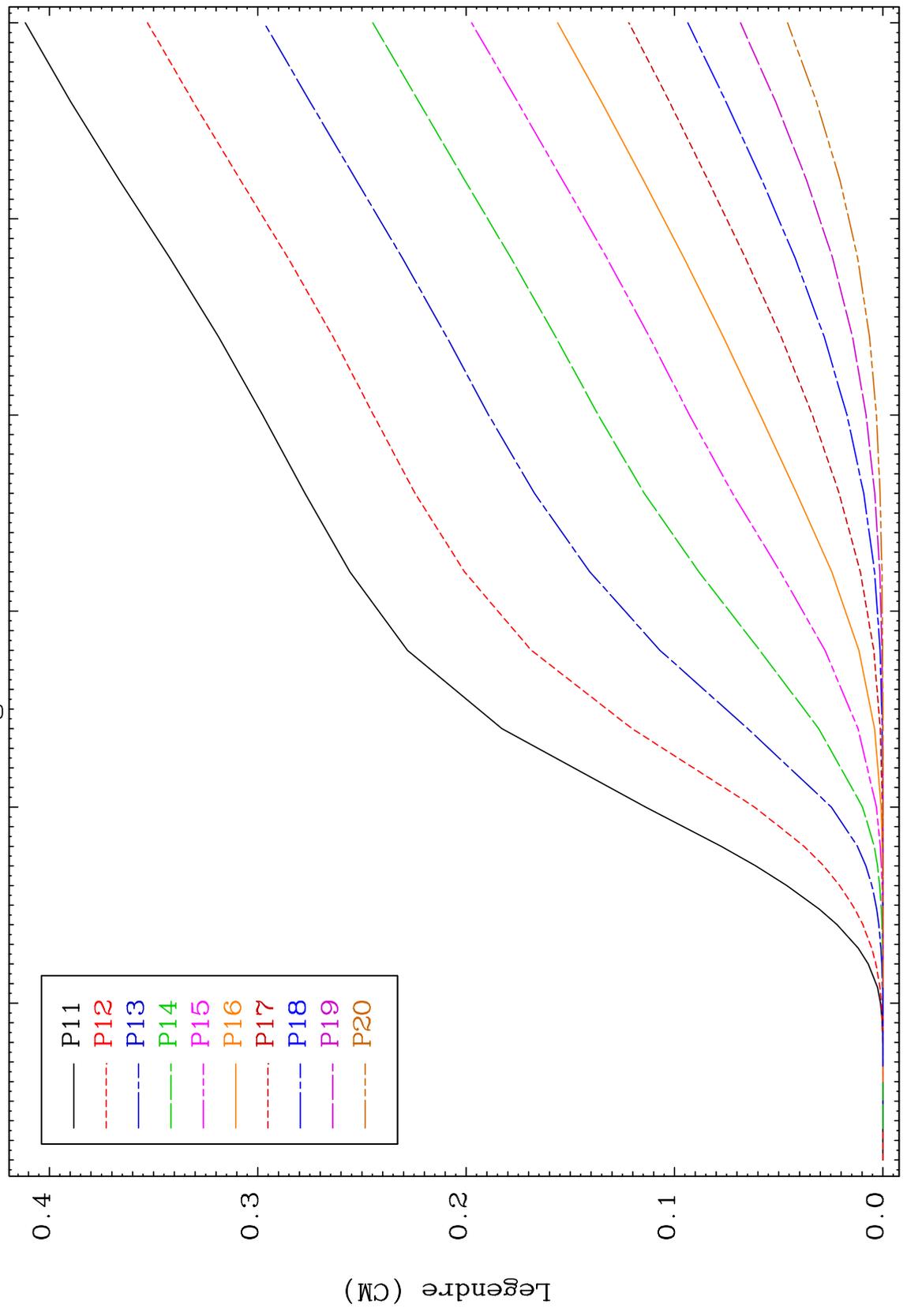
104-Rf-257



MAT 457

Elastic Legendre Coefficients

104-Rf-257



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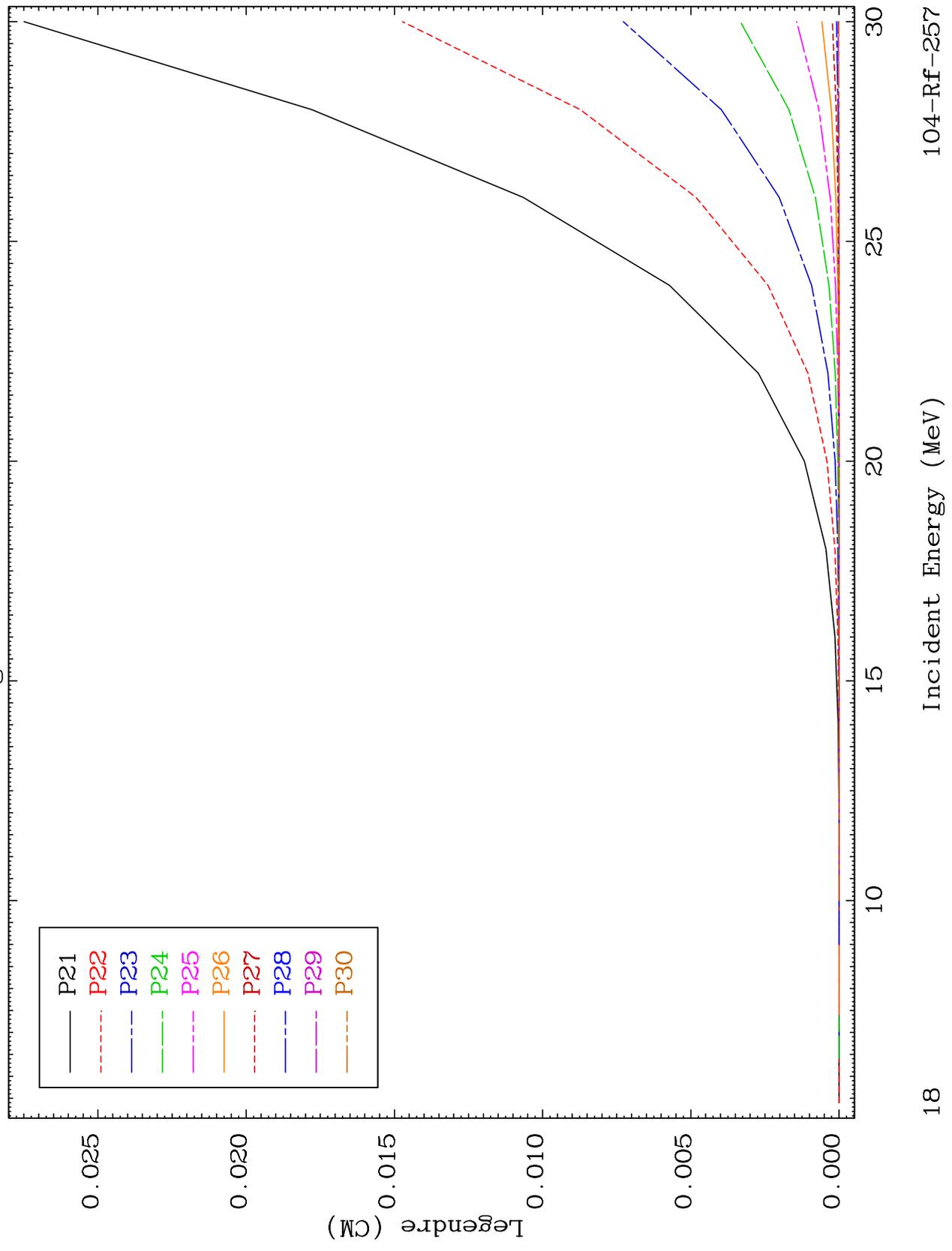
Incident Energy (MeV)

104-Rf-257

MAT 457

Elastic Legendre Coefficients

104-Rf-257



18

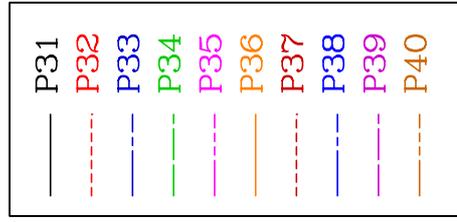
Incident Energy (MeV)

104-Rf-257

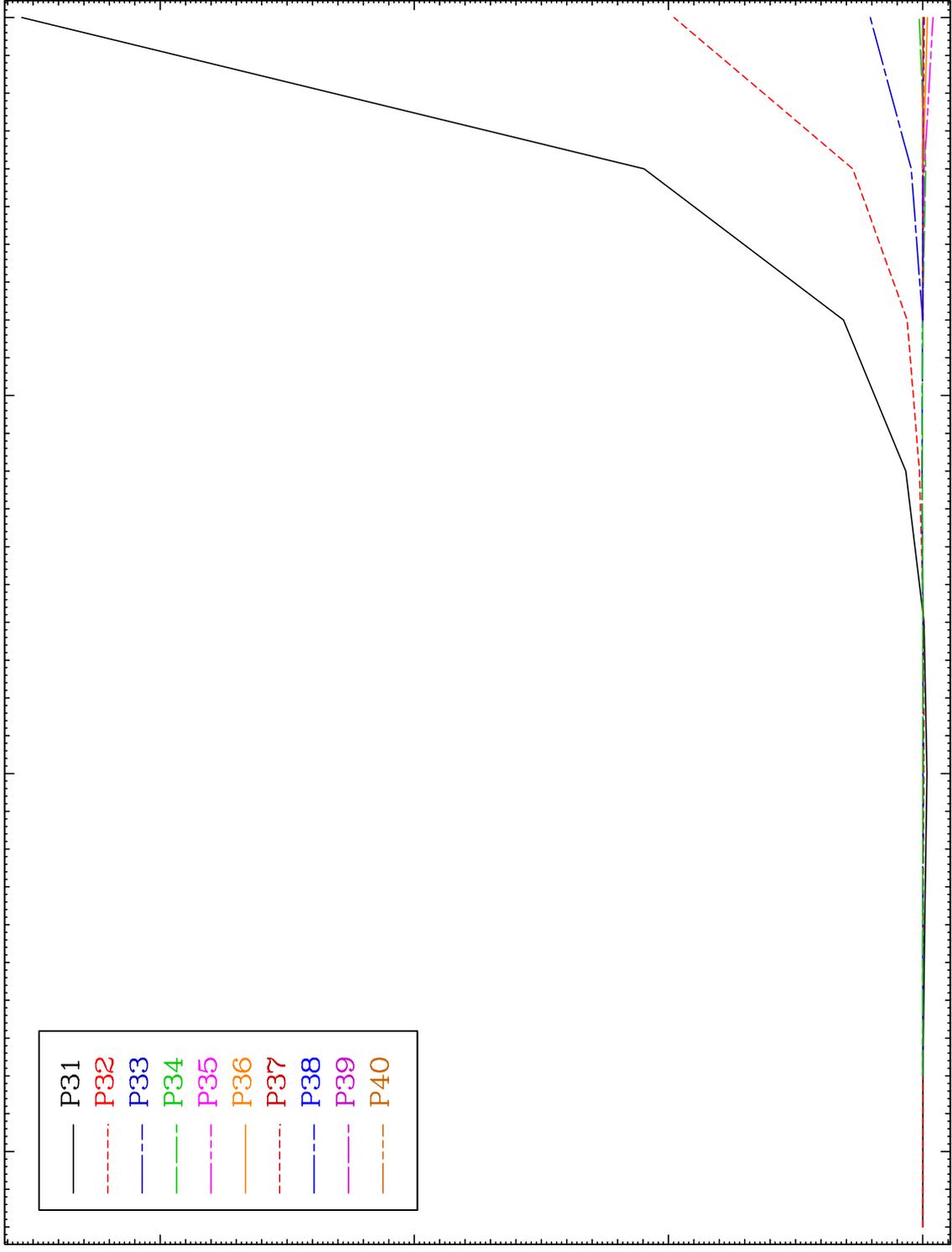
MAT 457

Elastic Legendre Coefficients

104-Rf-257



Legendre (CM) $\times 10^{-6}$

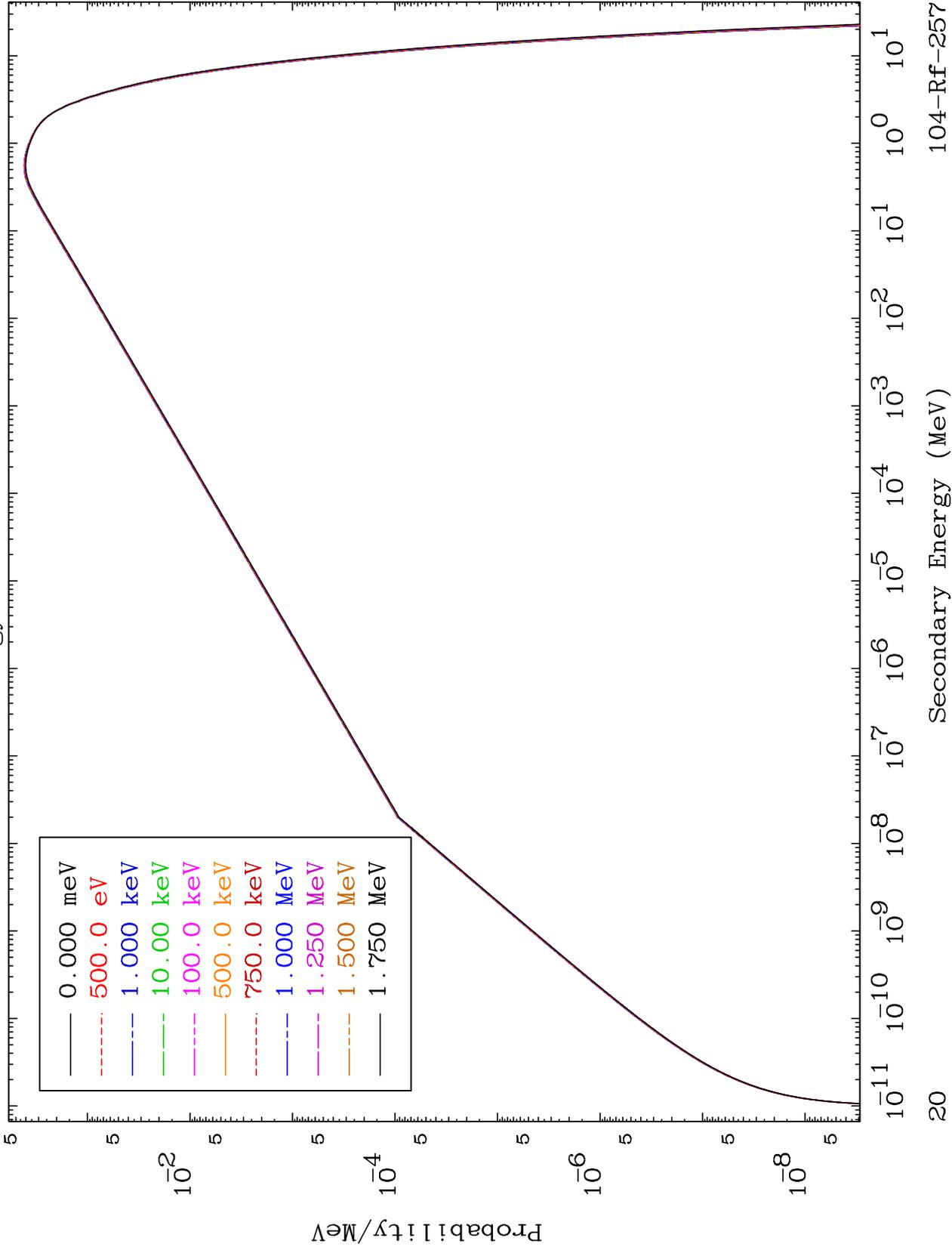


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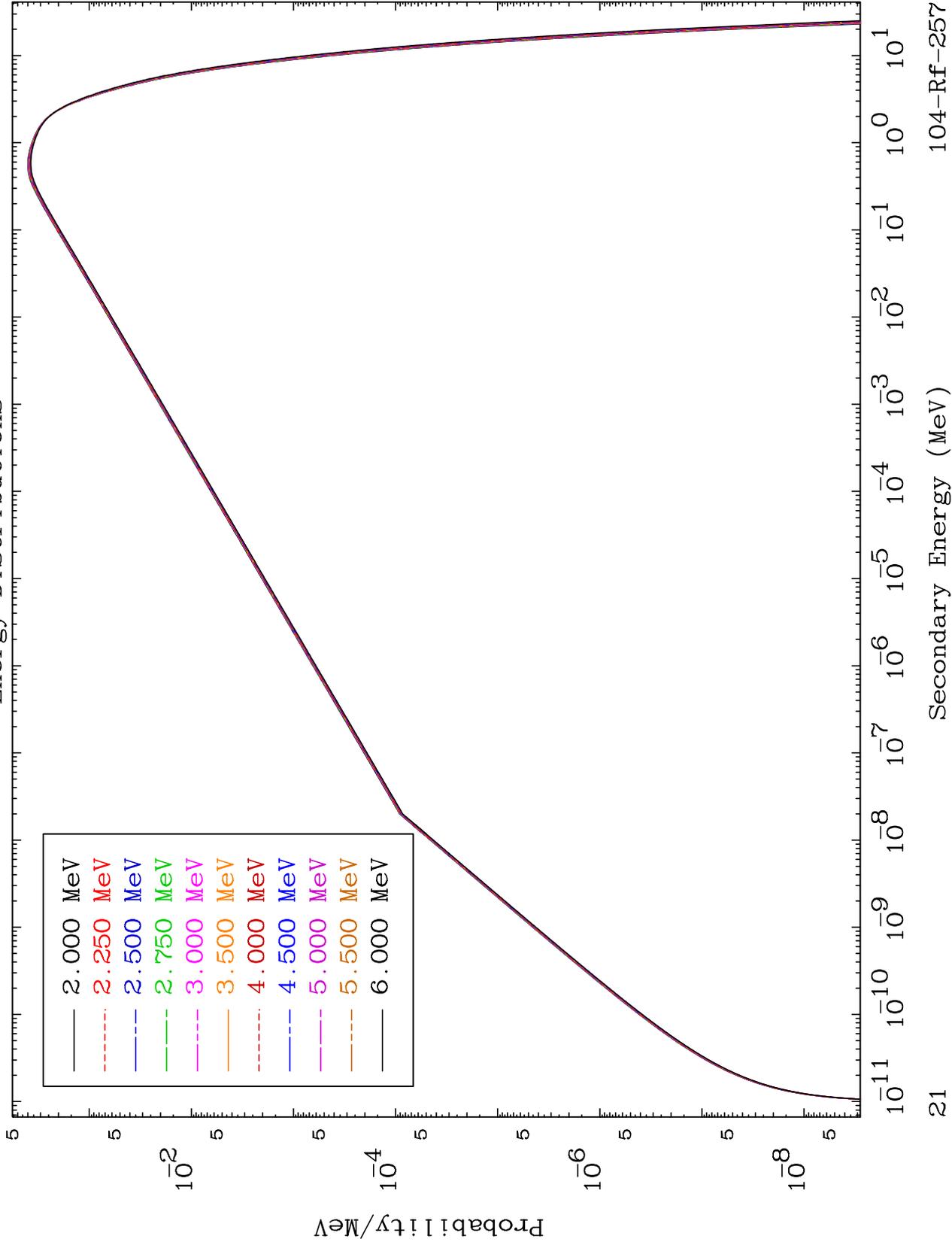
Incident Energy (MeV)

104-Rf-257

Fission Energy Distributions



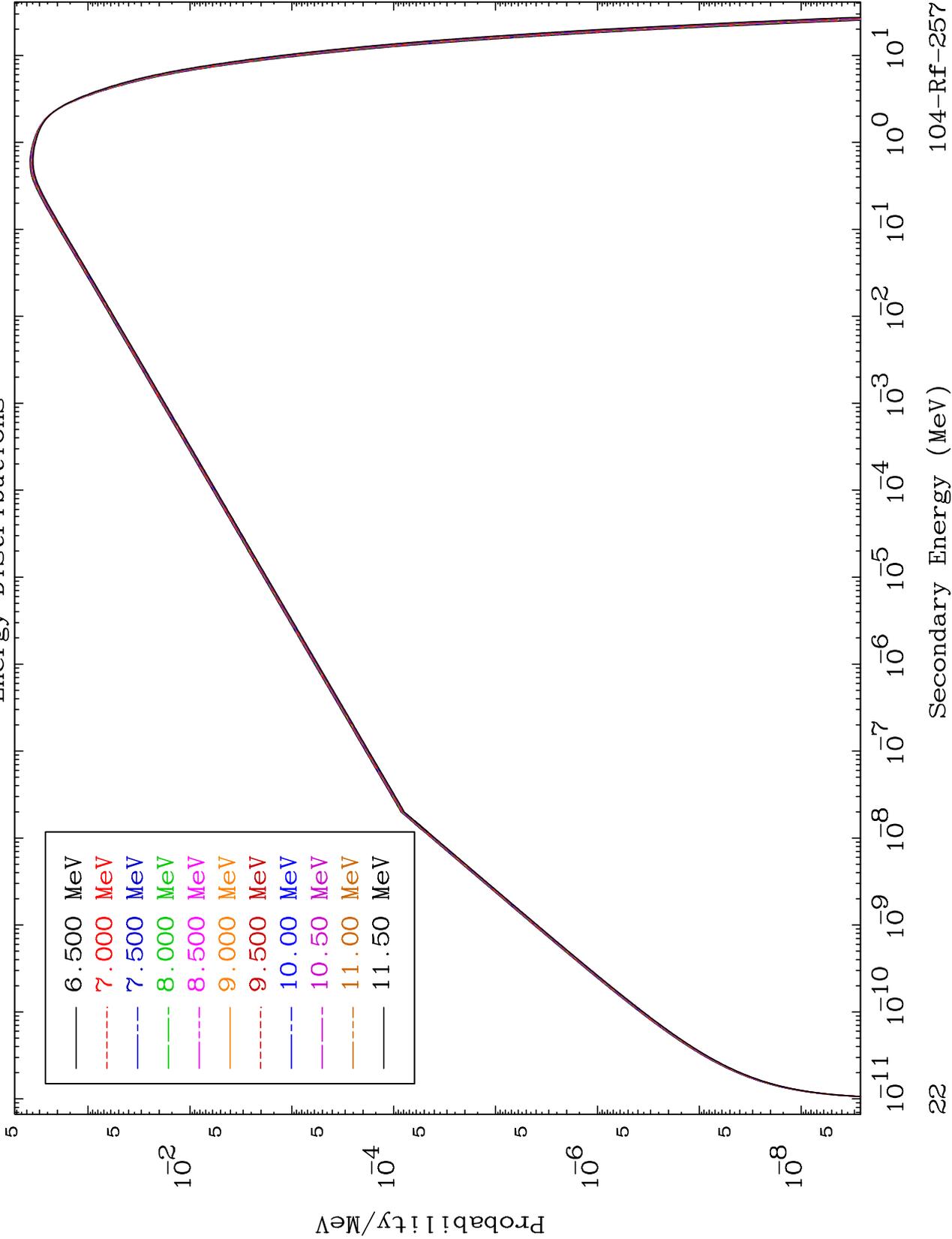
Fission Energy Distributions



MAT 457

Fission Energy Distributions

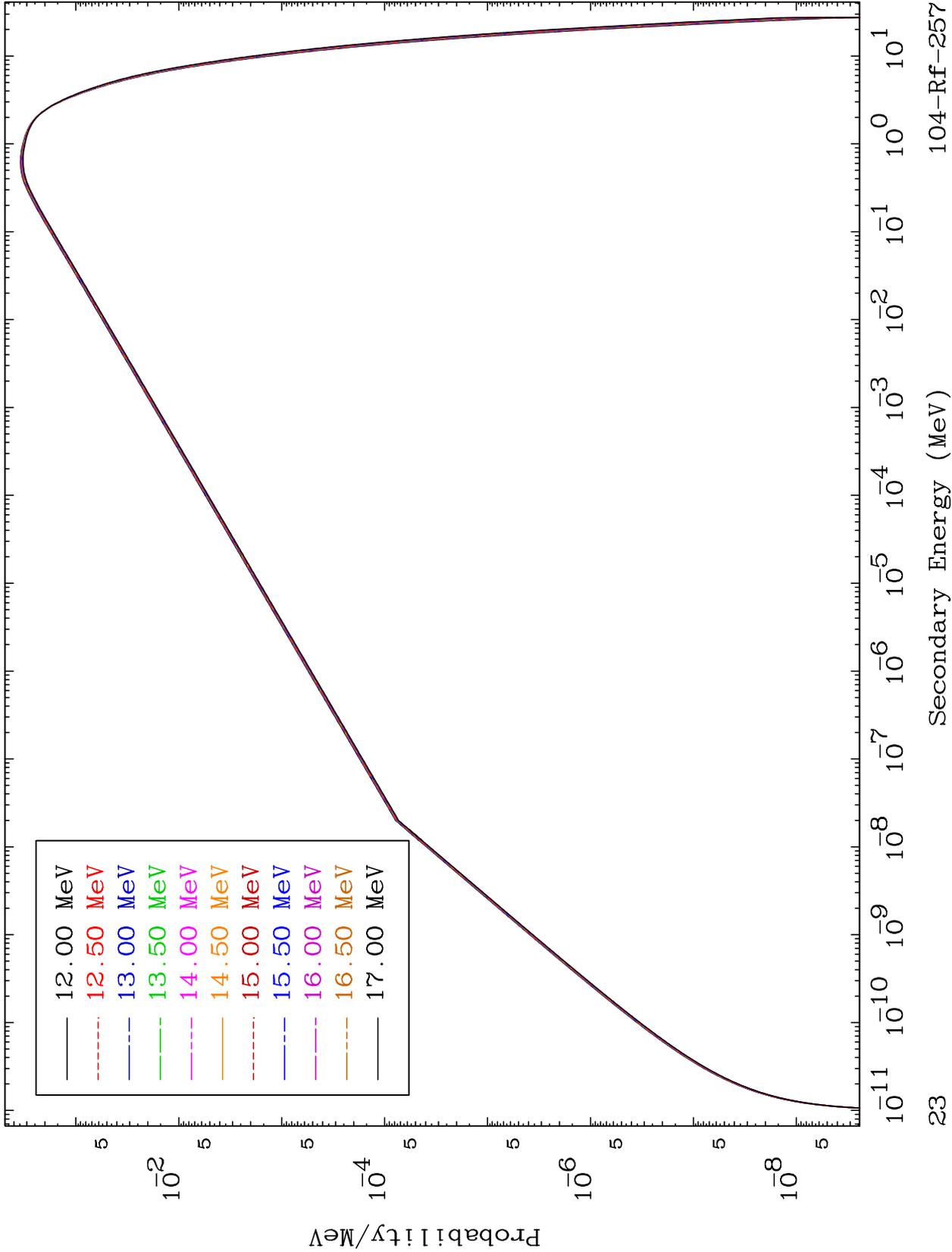
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MAT 457

Fission Energy Distributions

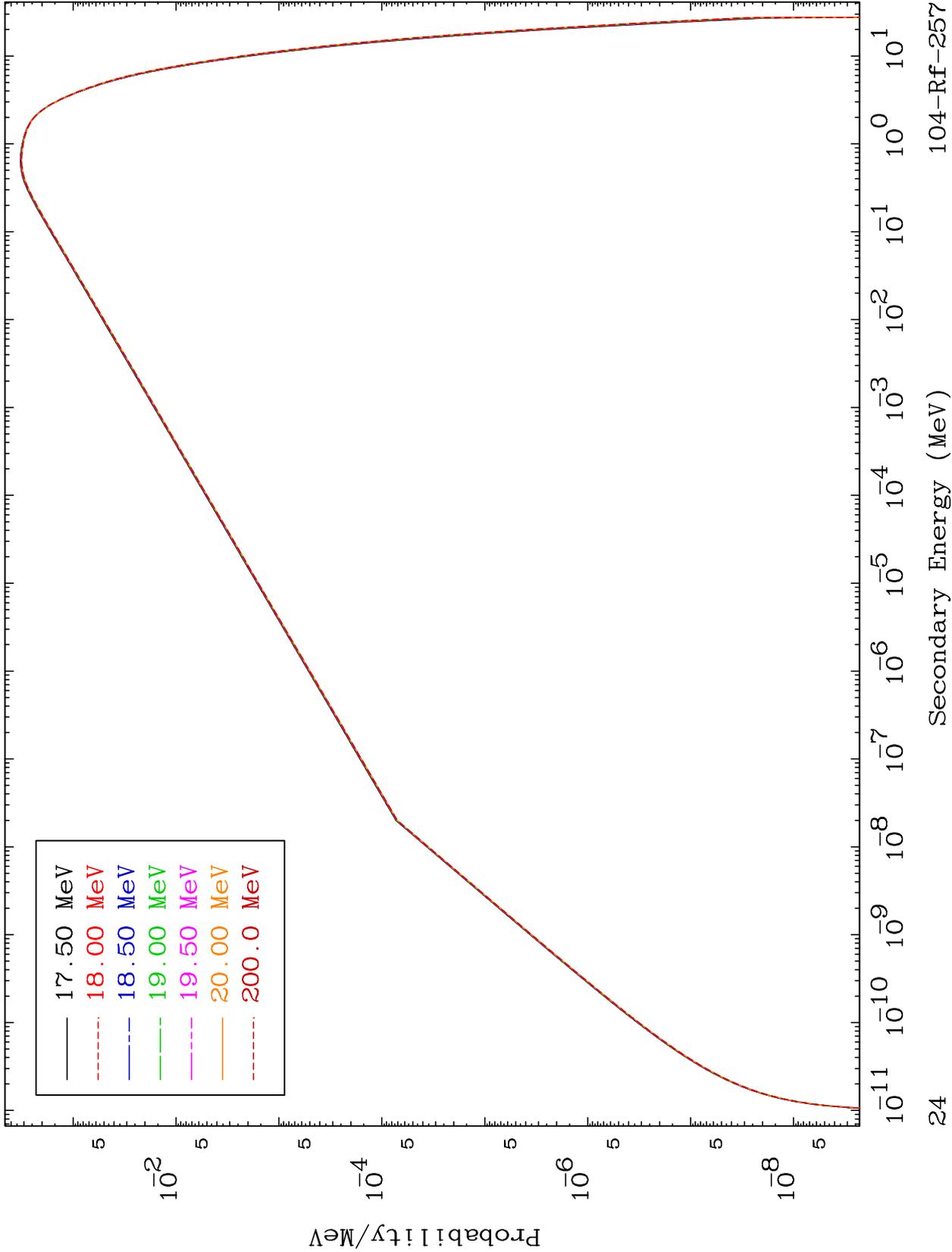
104-Rf-257



104-Rf-257

Secondary Energy (MeV)

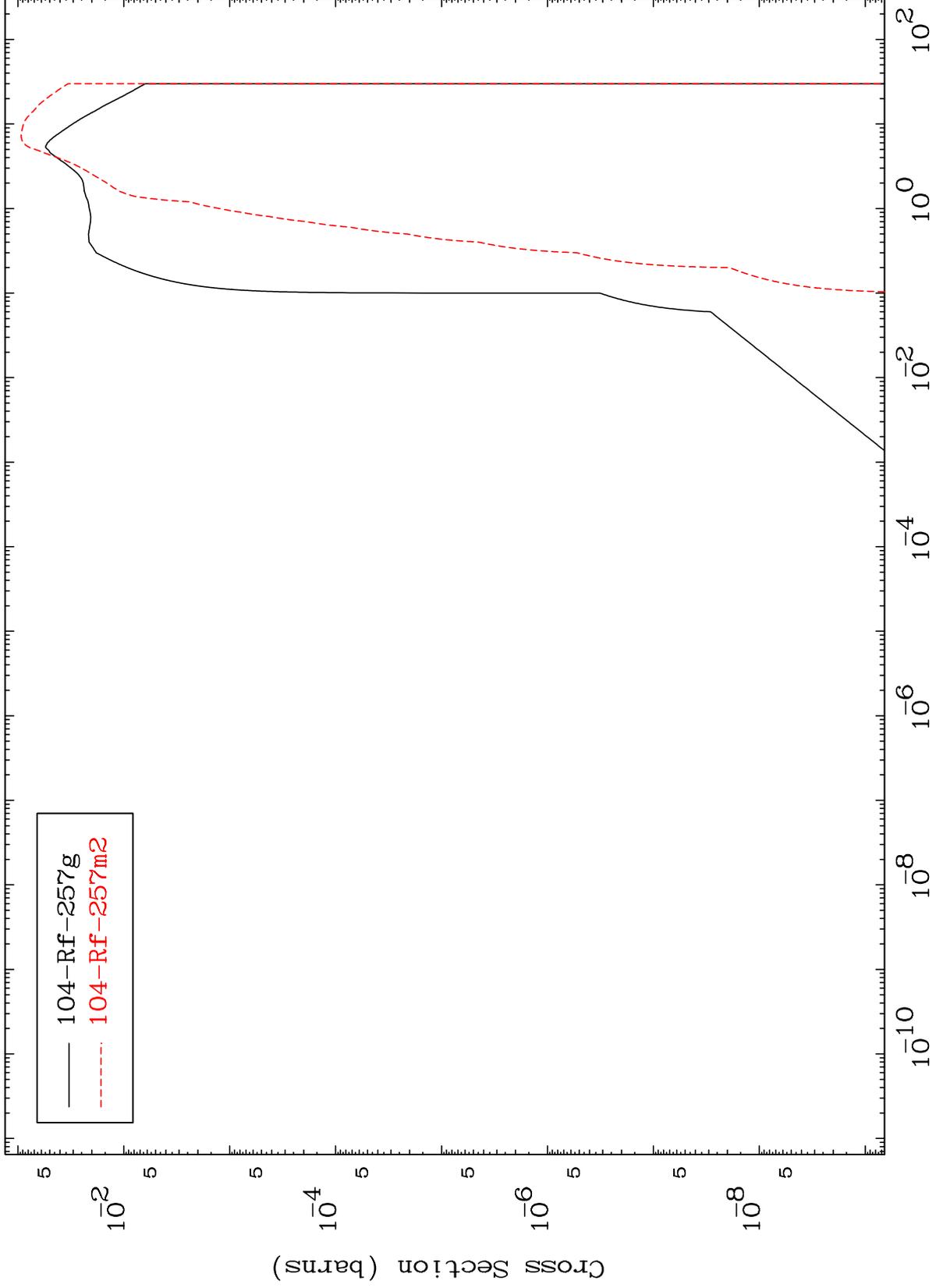
Fission Energy Distributions



MAT 457

Radionuclide Production Cross Section

104-Rf-257



25

Incident Energy (MeV)

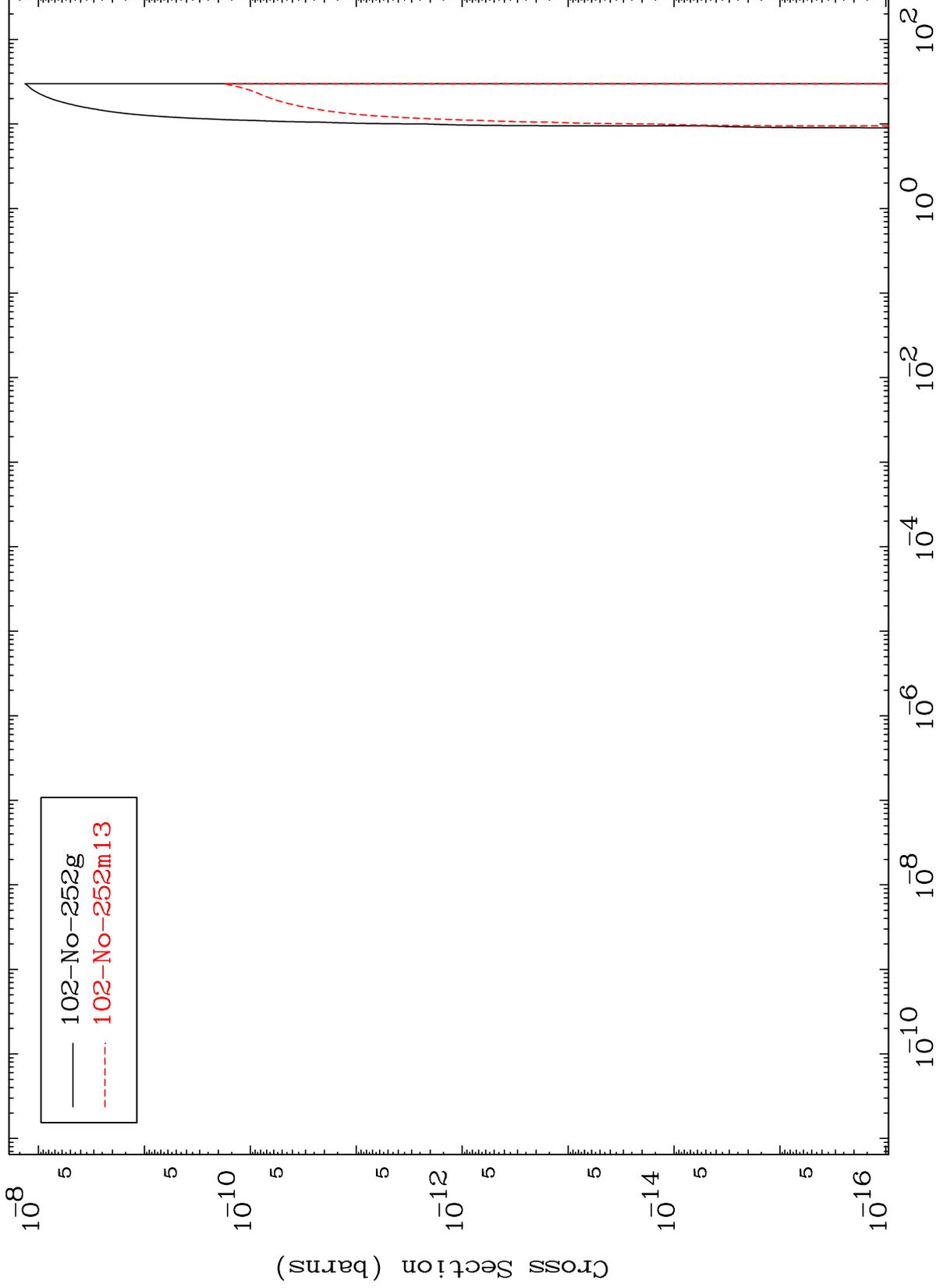
104-Rf-257

MAT 457

(n,2n) α

104-Rf-257

Radionuclide Production Cross Section



26

Incident Energy (MeV)

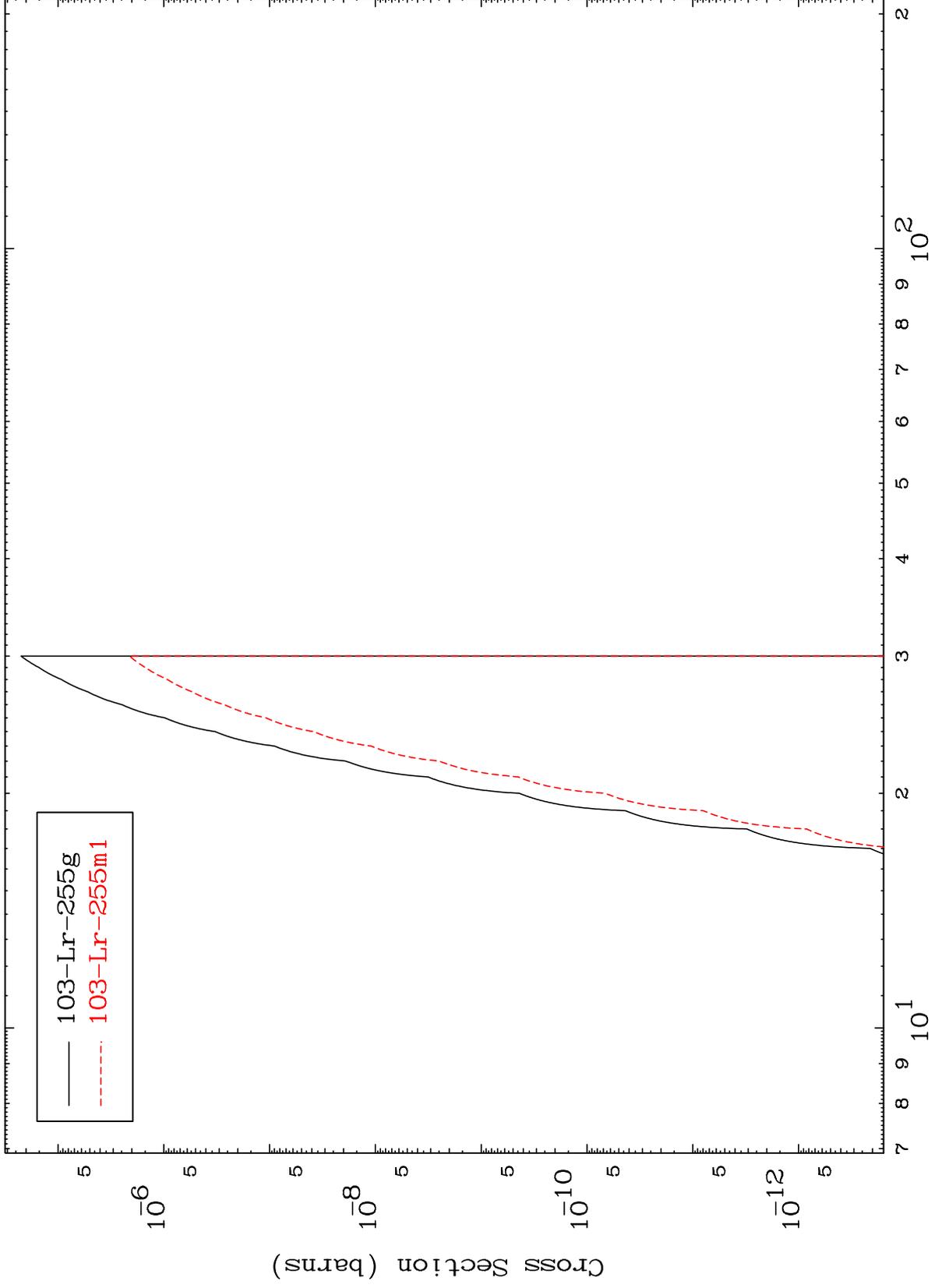
104-Rf-257

MAT 457

(n,n') d

104-Rf-257

Radionuclide Production Cross Section



27

Incident Energy (MeV)

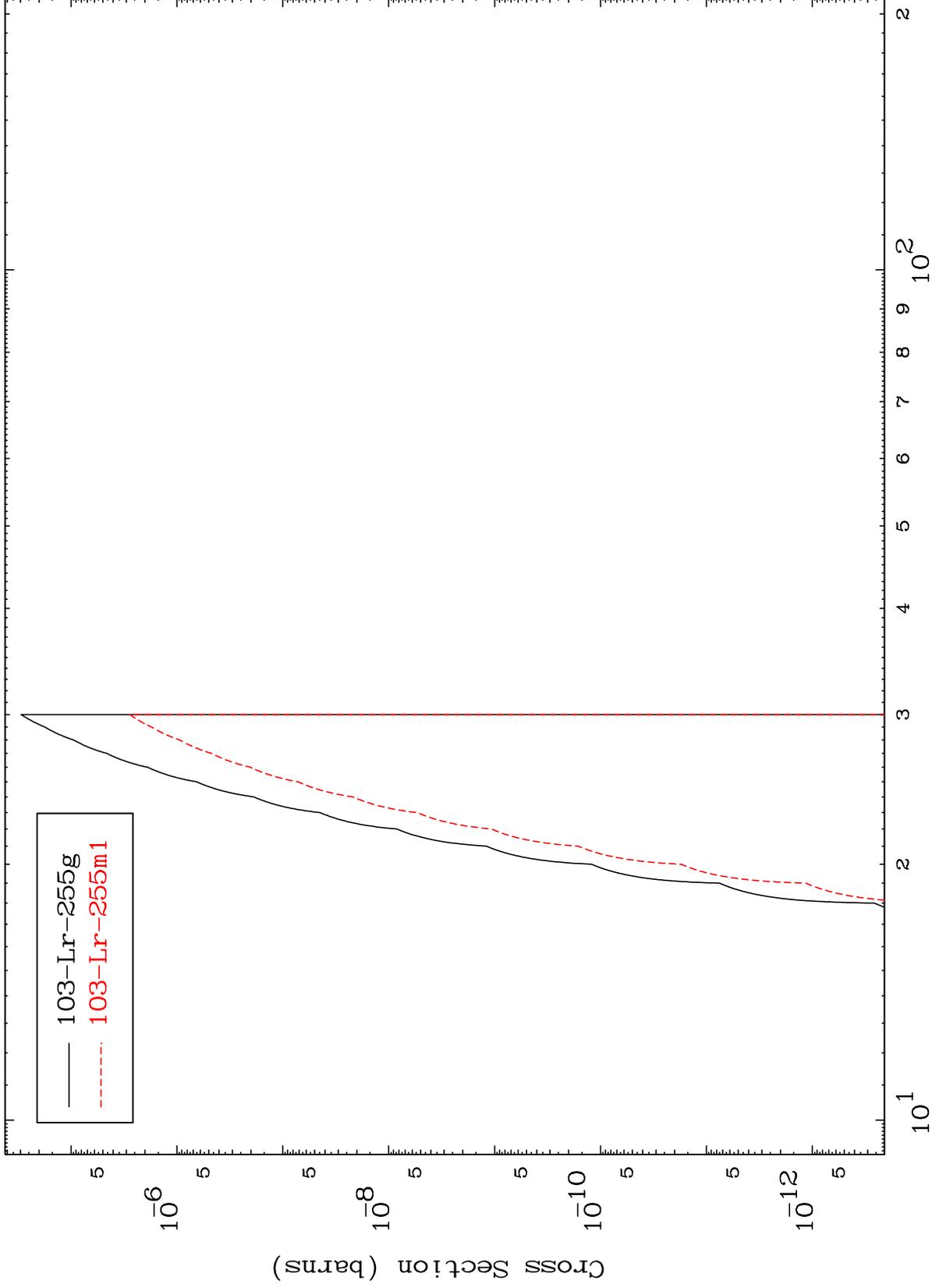
104-Rf-257

MAT 457

(n,2n) p

104-Rf-257

Radionuclide Production Cross Section



28

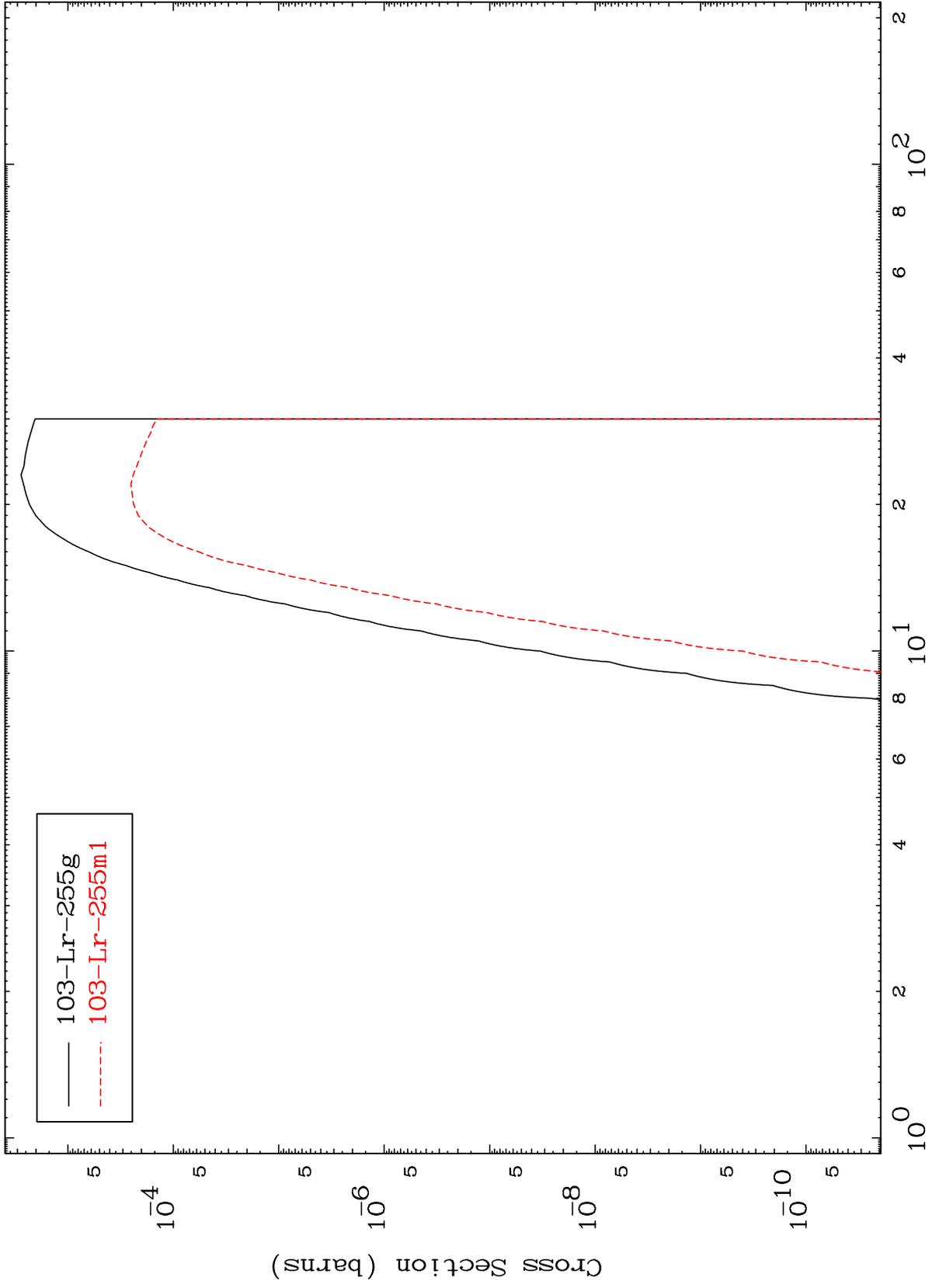
Incident Energy (MeV)

104-Rf-257

MAT 457

104-Rf-257

(n, t)
Radionuclide Production Cross Section



29

Incident Energy (MeV)

104-Rf-257

MAT 457

104-Rf-257

Radionuclide Production Cross Section
(n, α)

