

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

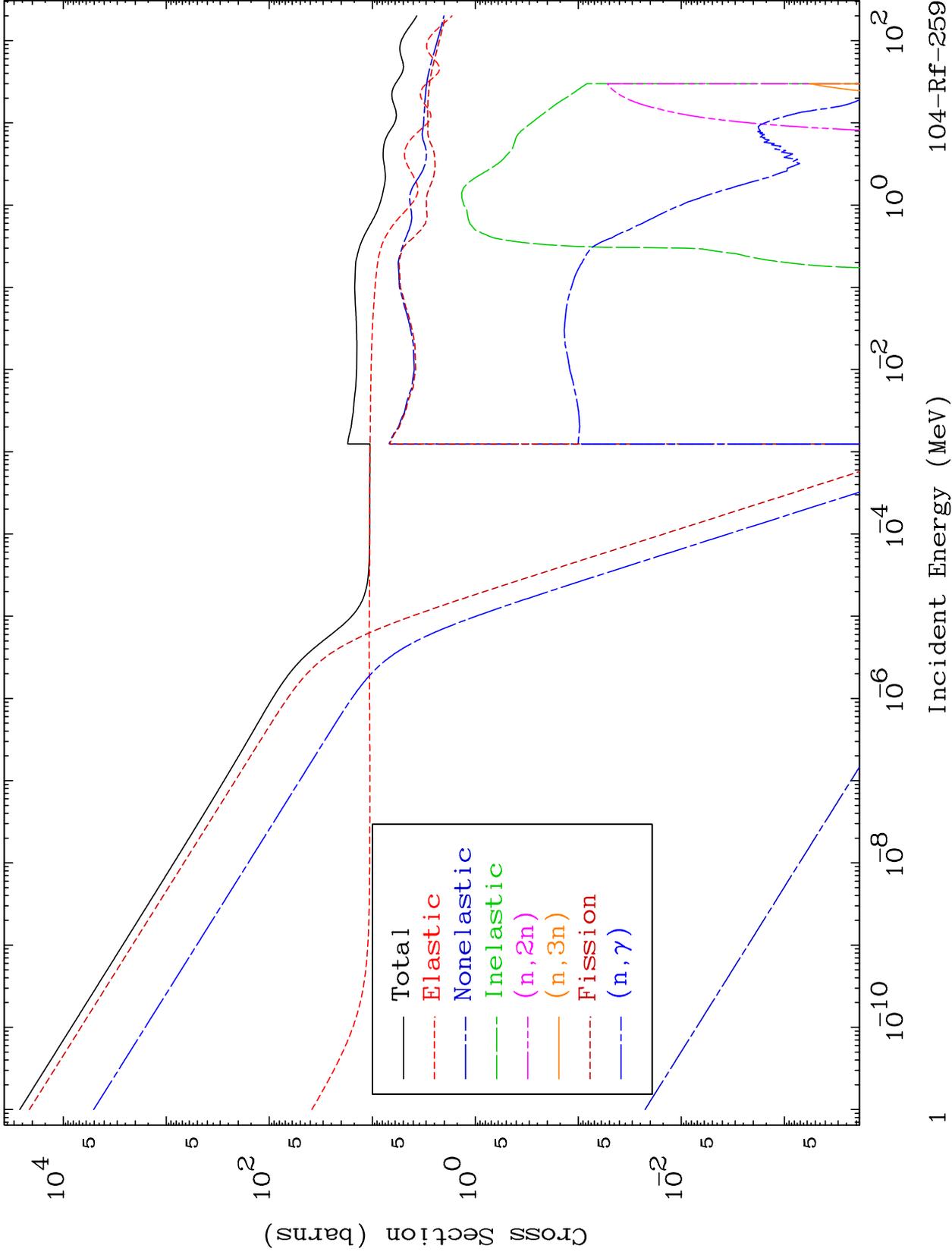
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Press Mouse Button to Start

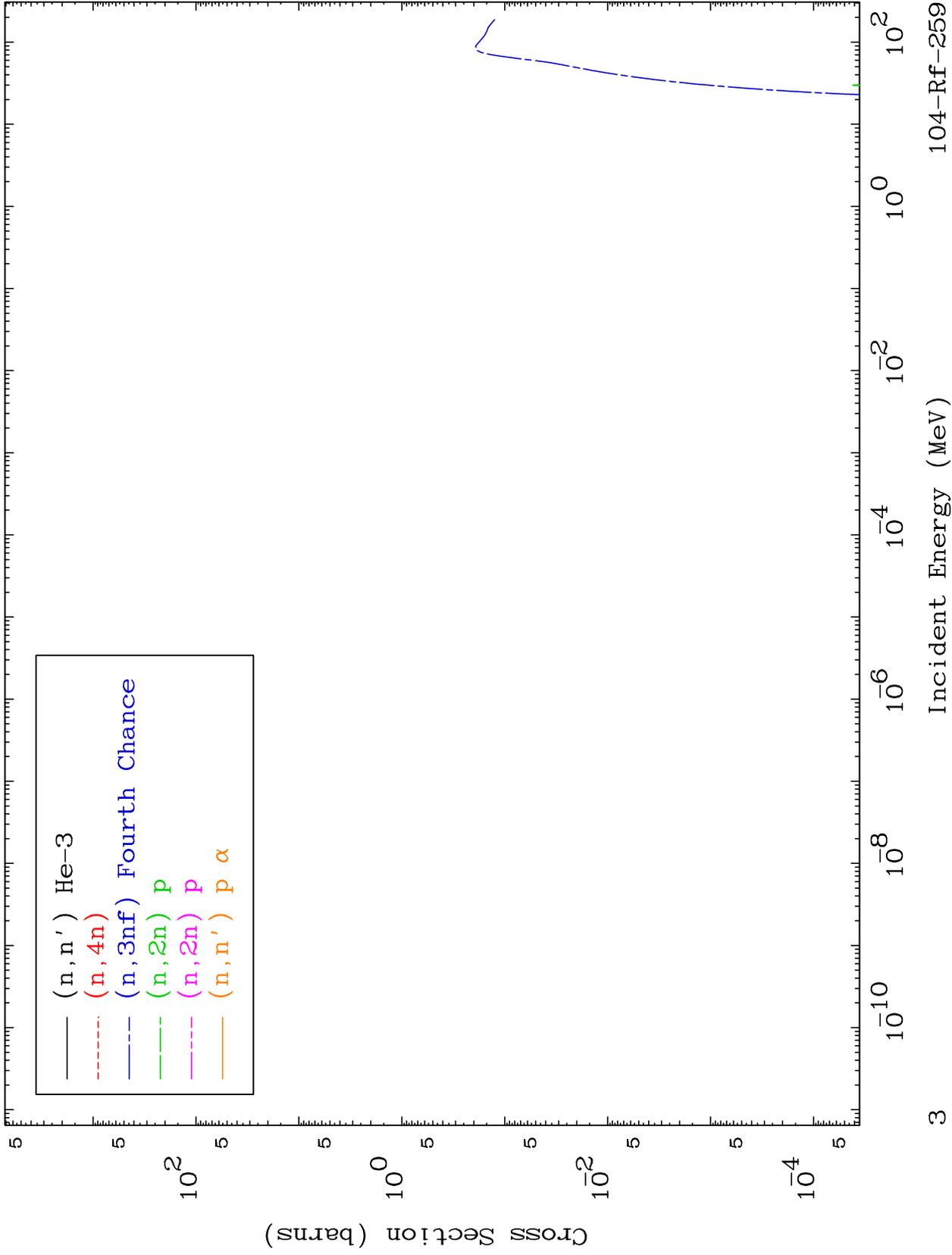
MAT 459

Neutron Major  
293 Kelvin Cross Sections

104-Rf-259



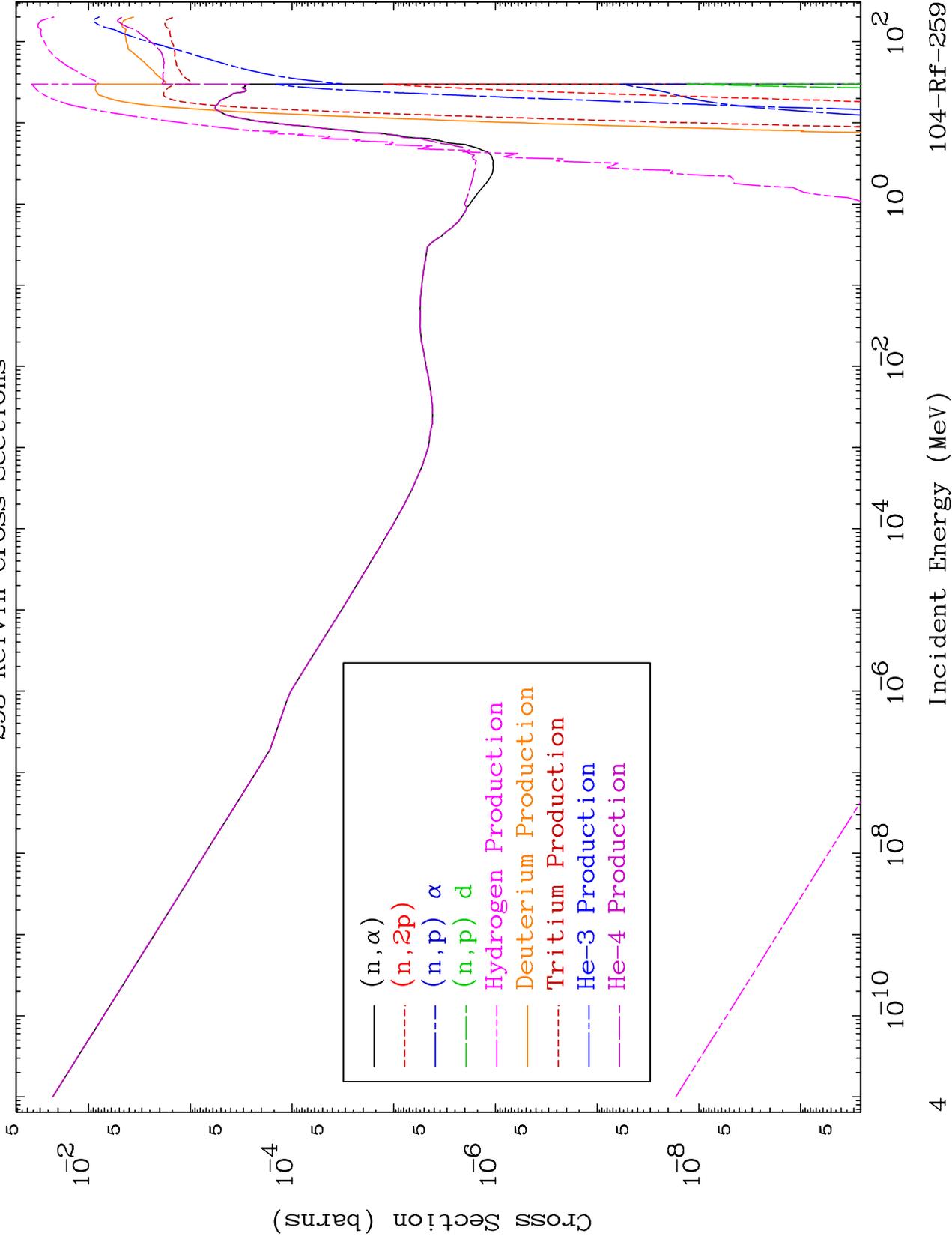




MAT 459

Neutron Absorption  
293 Kelvin Cross Sections

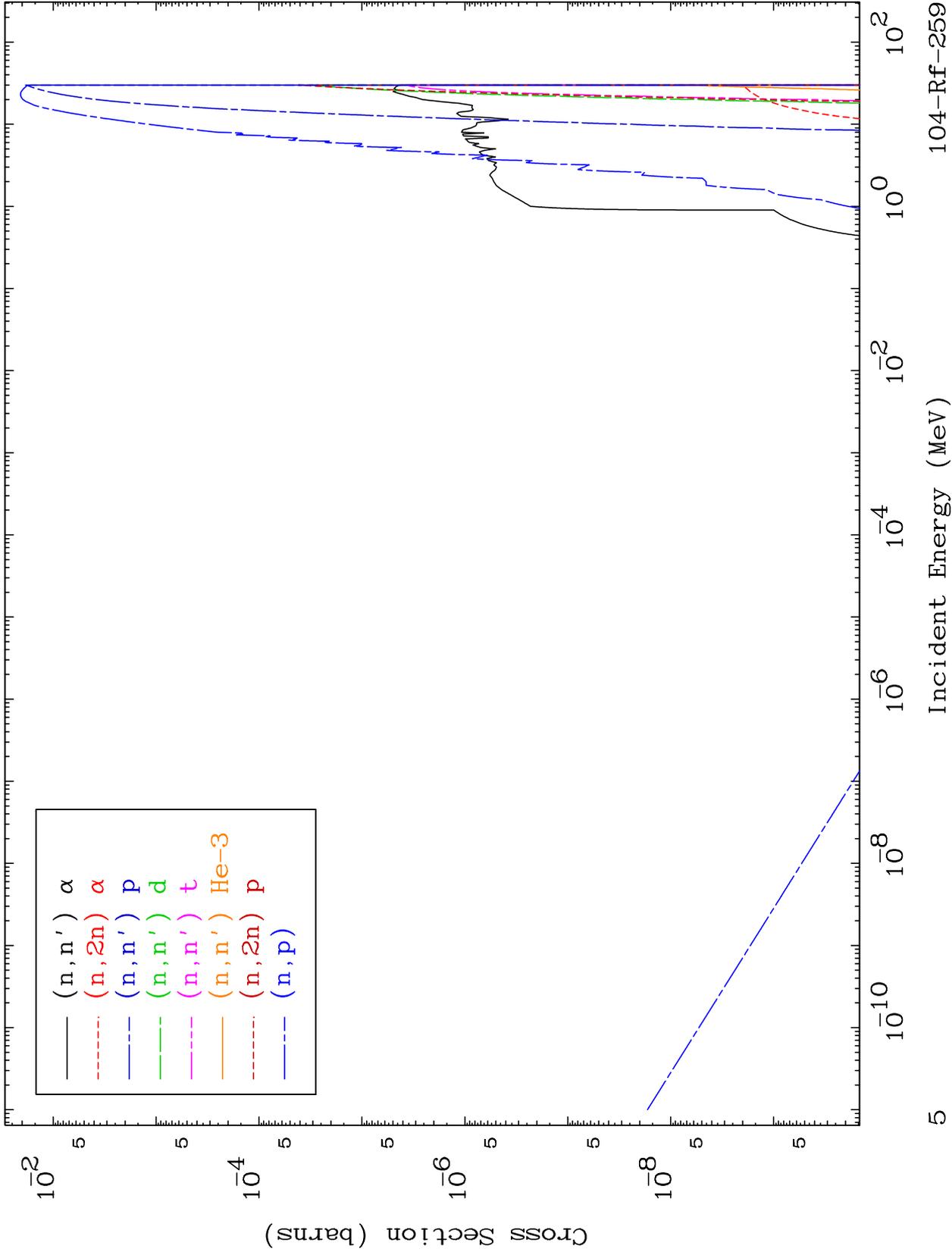
104-Rf-259



MAT 459

Charged Particle  
293 Kelvin Cross Sections

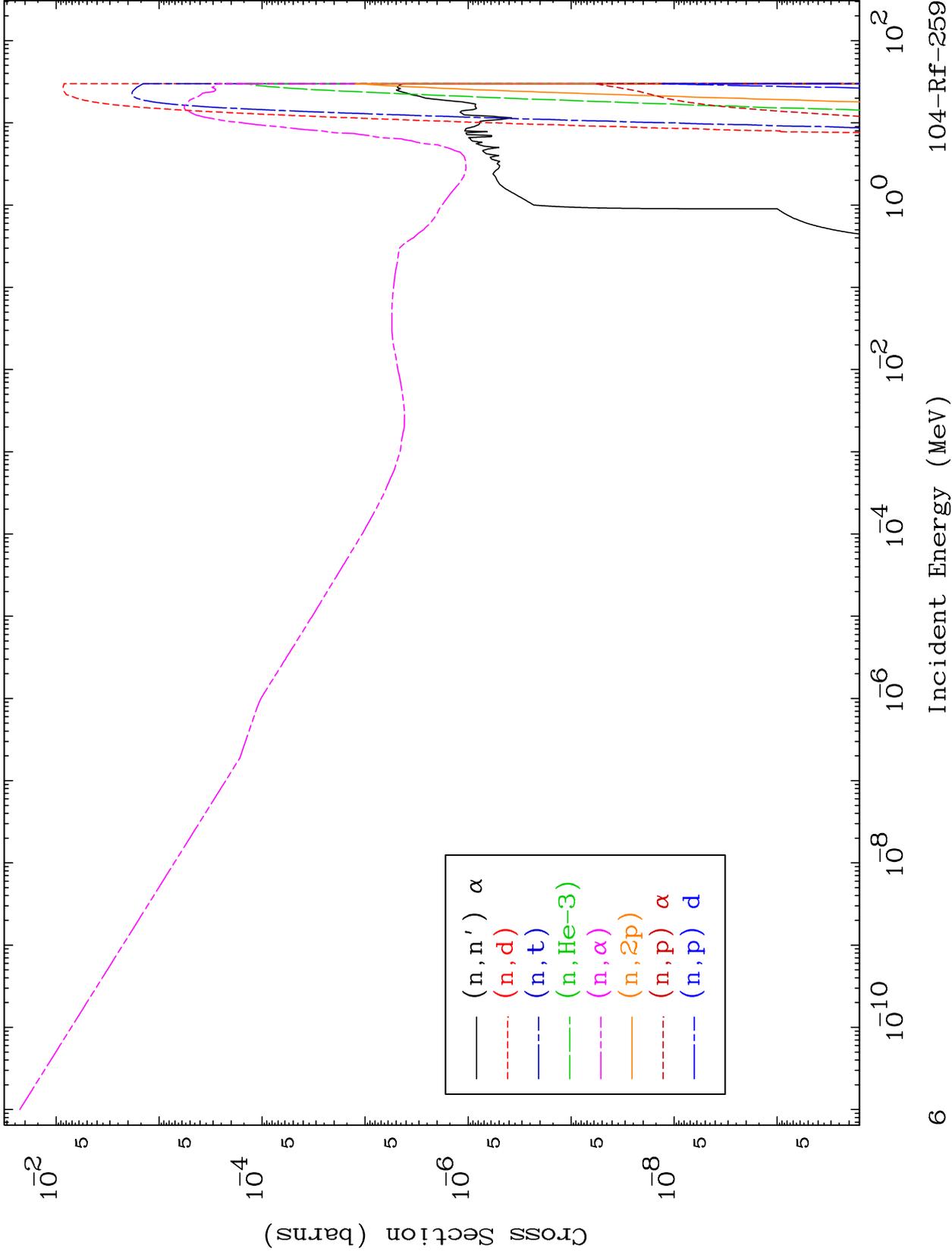
104-Rf-259



MAT 459

Charged Particle  
293 Kelvin Cross Sections

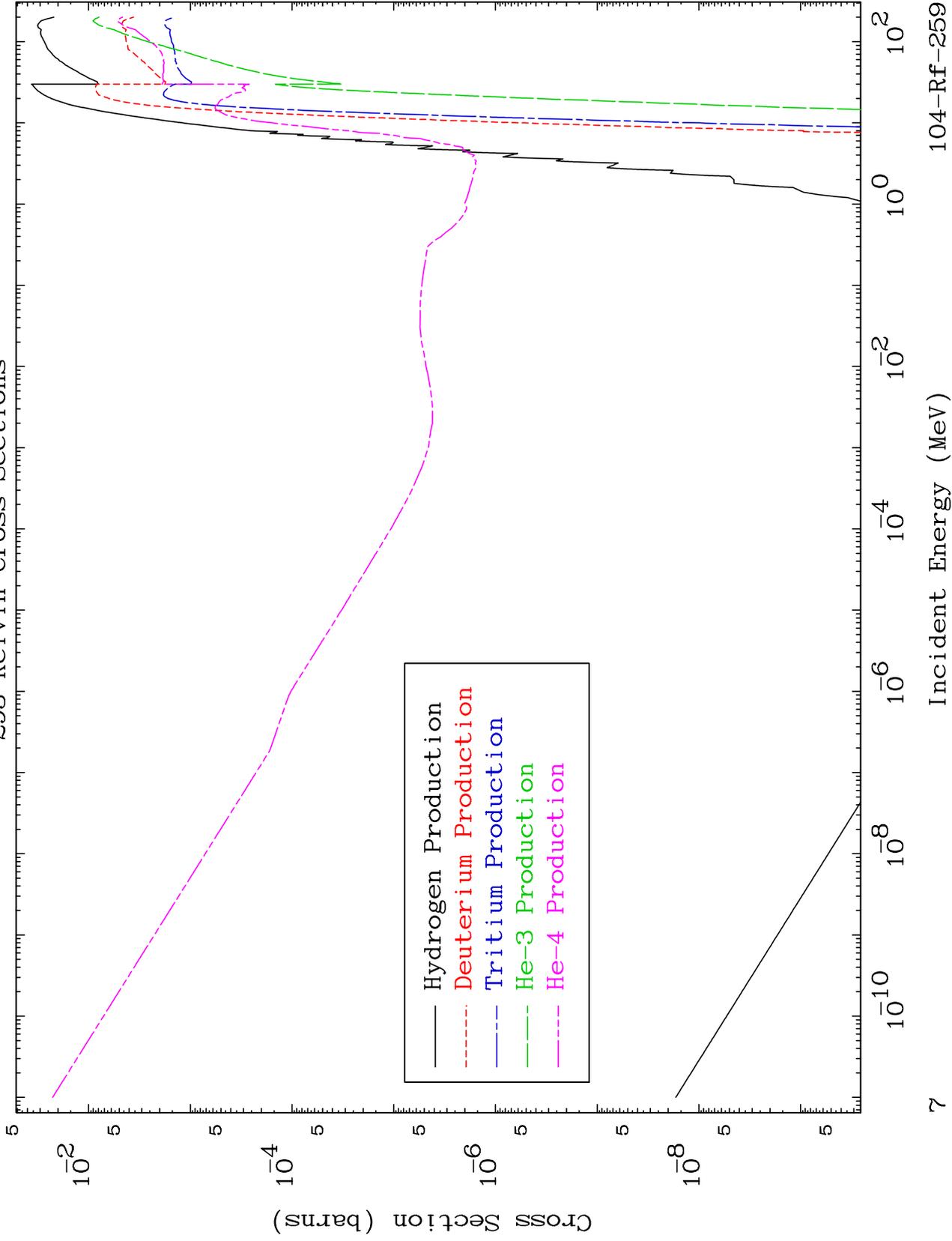
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MAT 459

Particle Production  
293 Kelvin Cross Sections

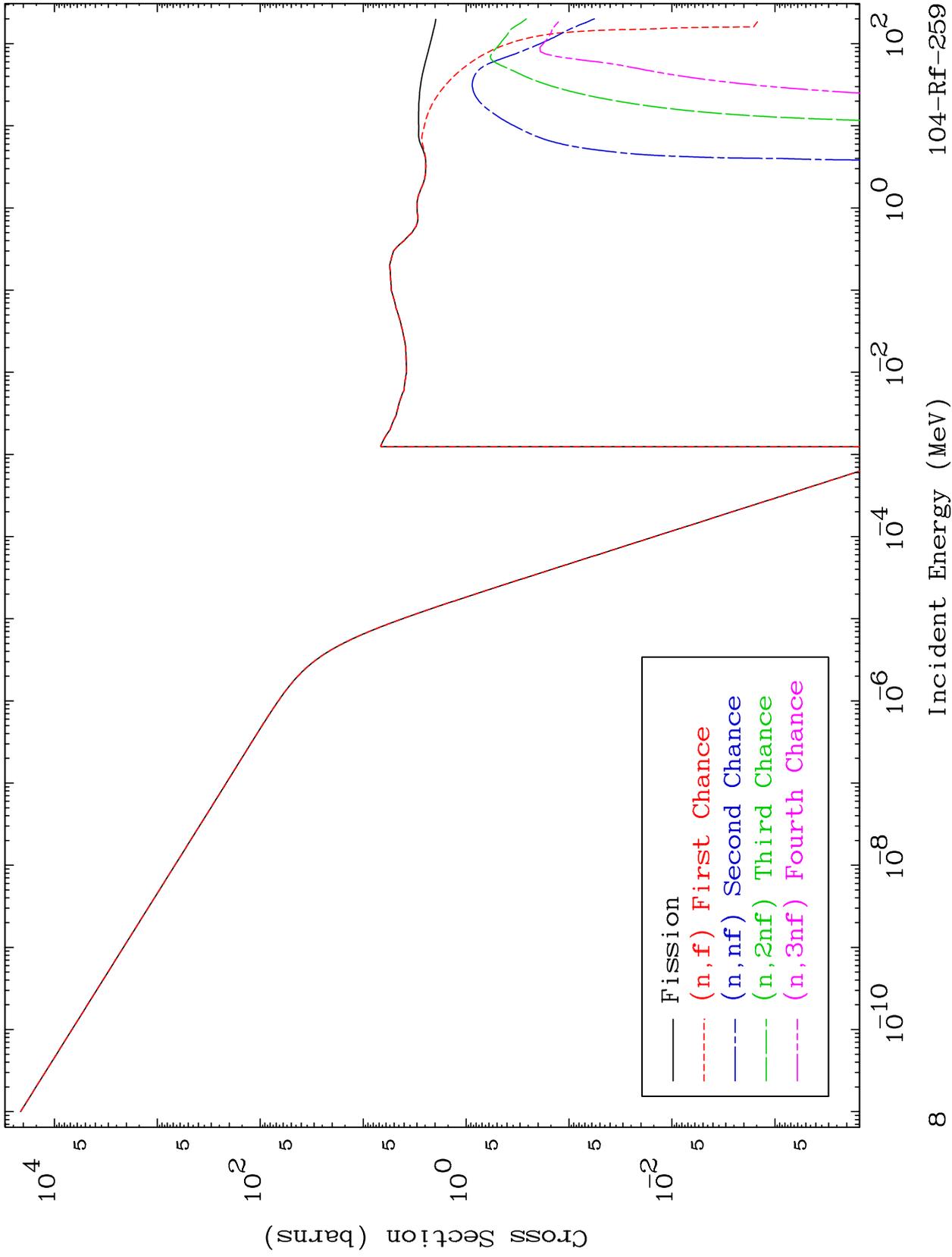
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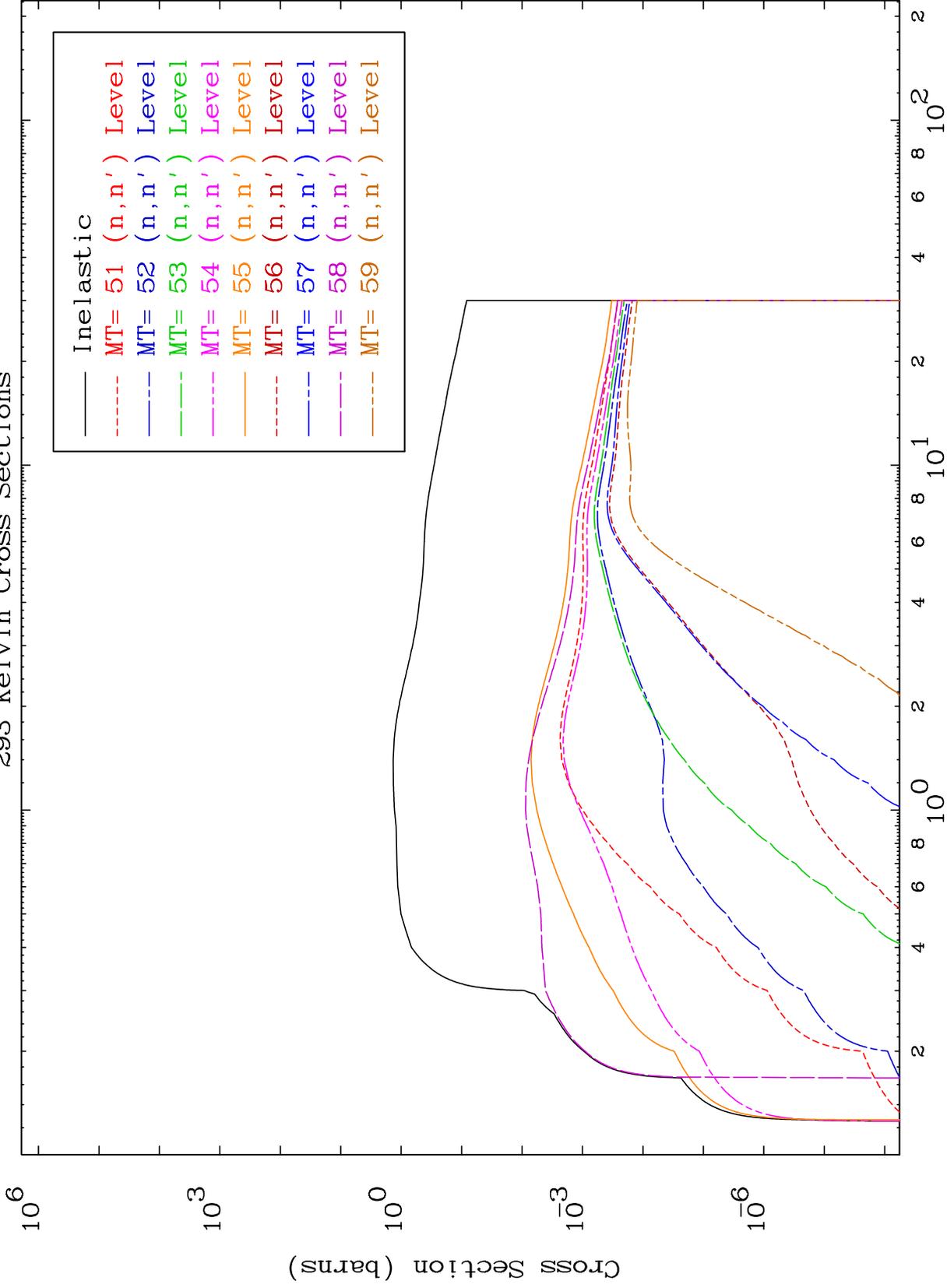
MAT 459

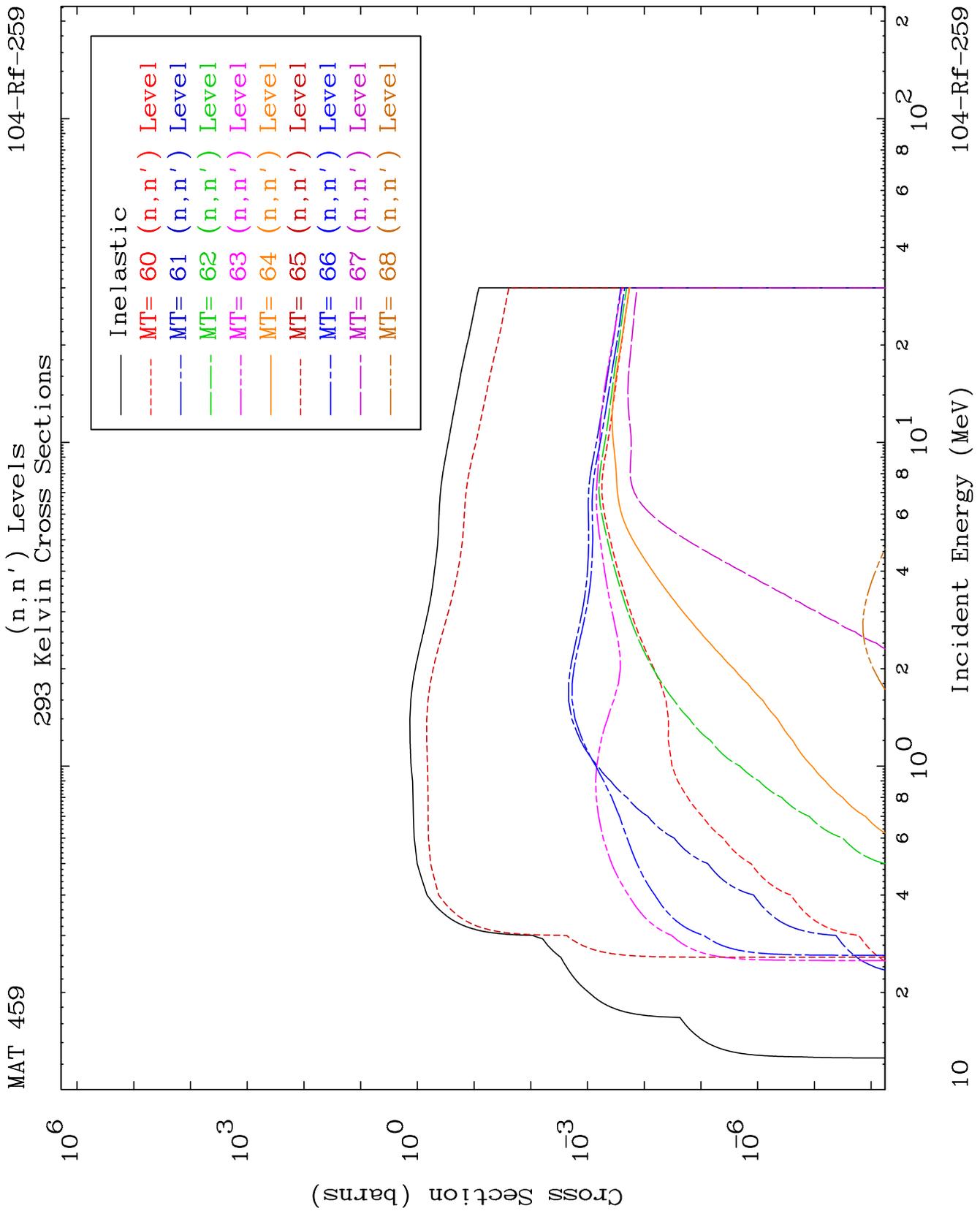
Fission  
293 Kelvin Cross Sections

104-Rf-259



293 Kelvin Cross Sections





MAT 459

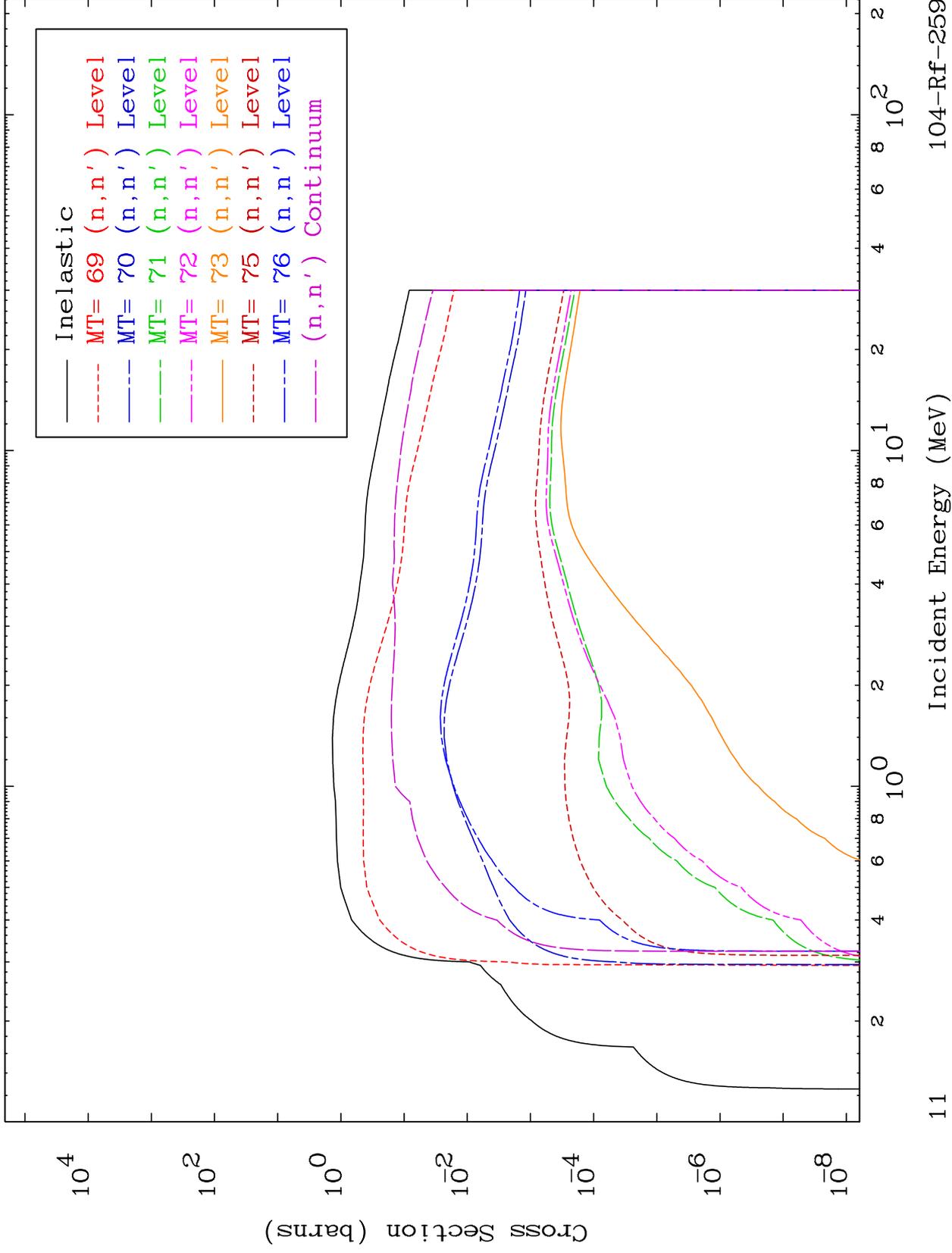
(n,n') Levels  
293 Kelvin Cross Sections

104-Rf-259

10

Incident Energy (MeV)

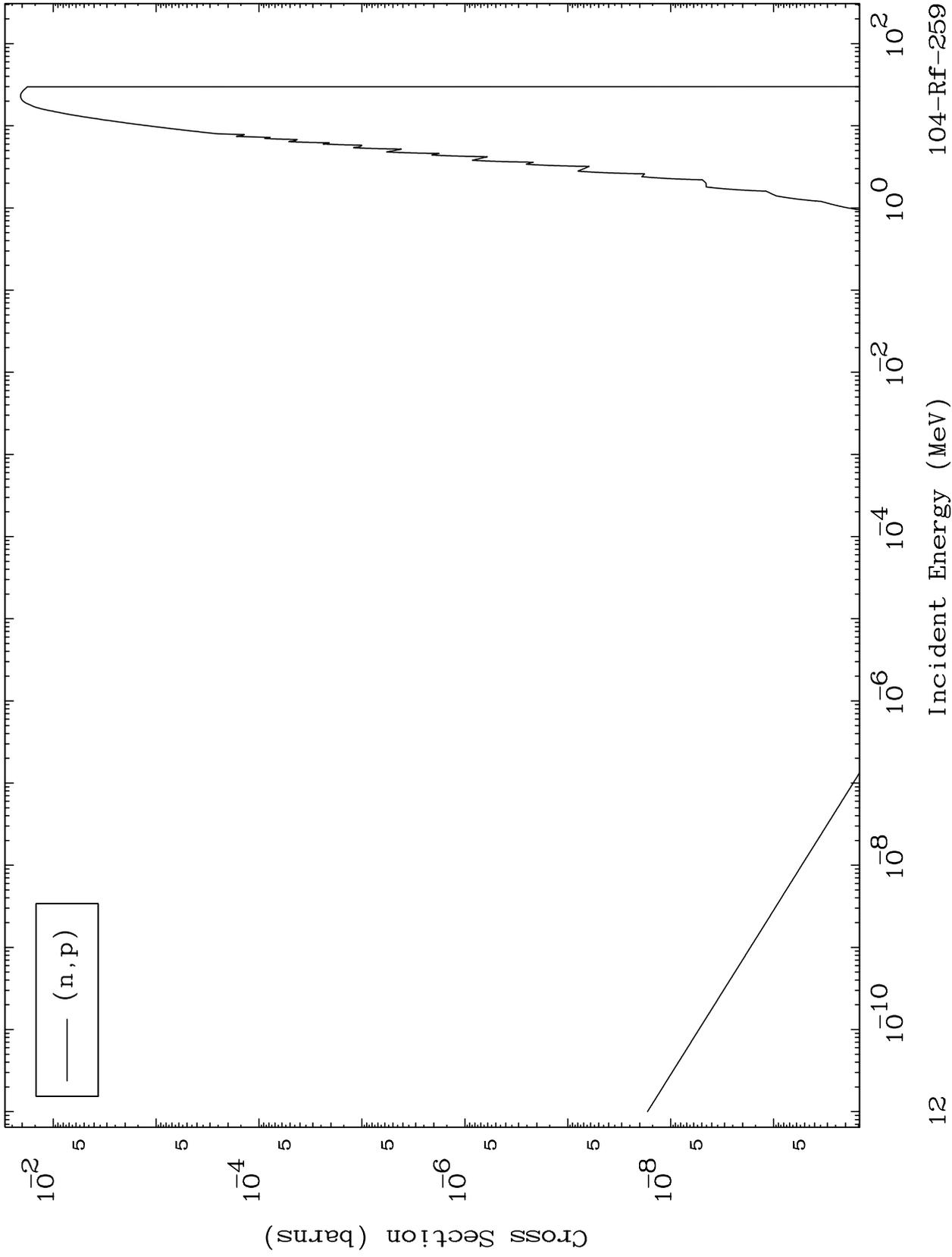
104-Rf-259



MAT 459

(n,p) Levels  
293 Kelvin Cross Sections

104-Rf-259

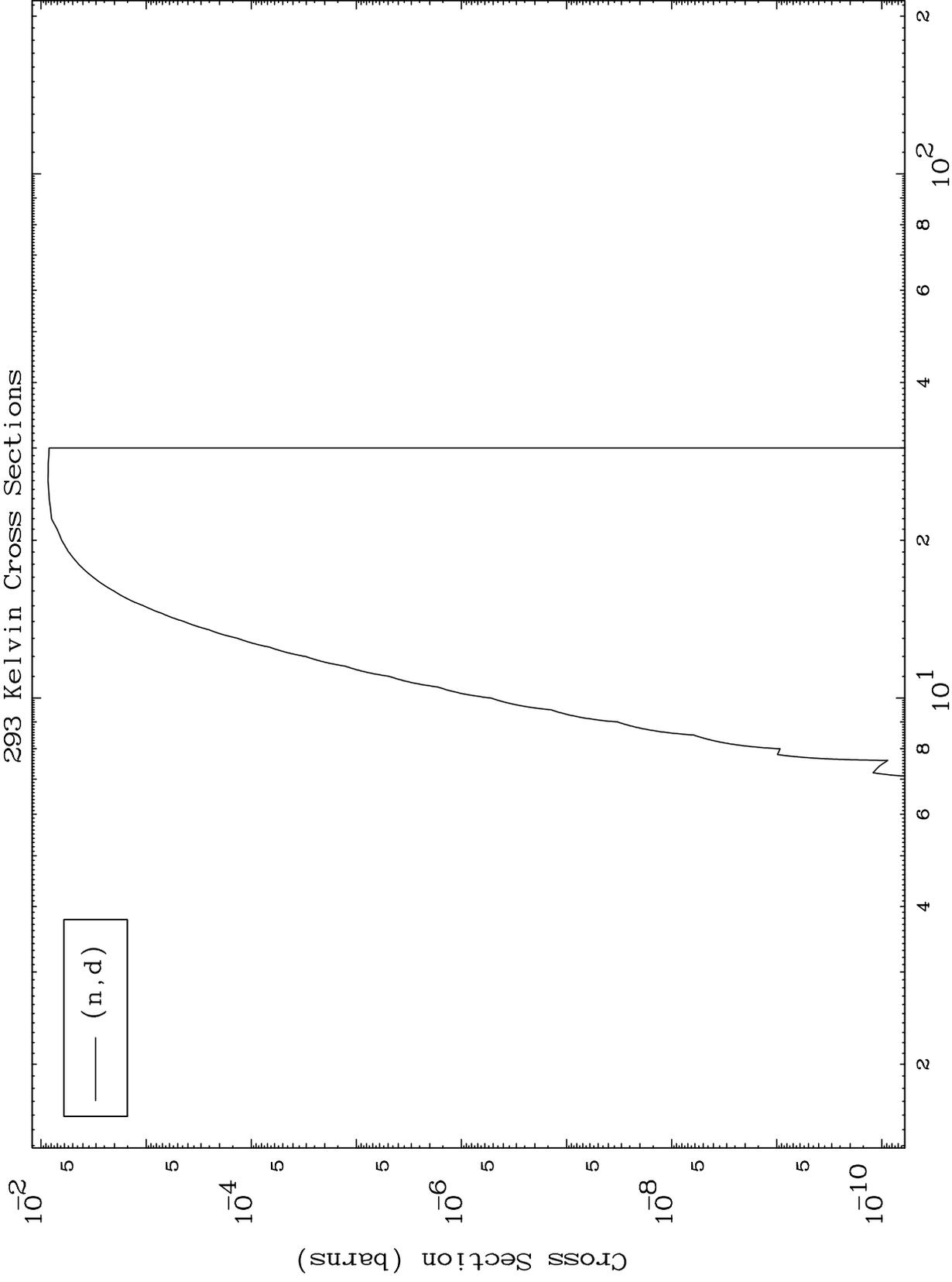


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(n,d) Levels  
293 Kelvin Cross Sections

104-Rf-259



13

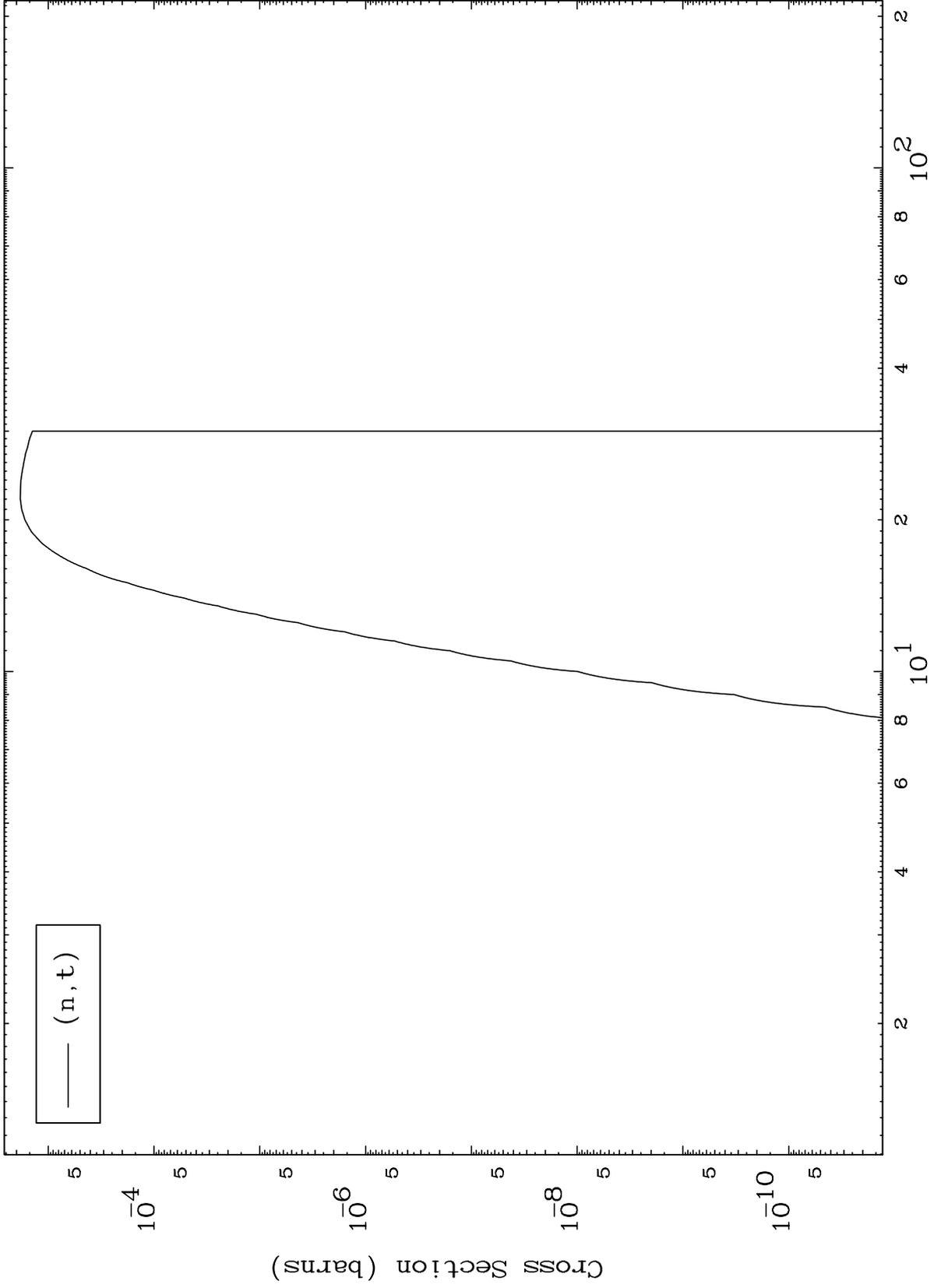
Incident Energy (MeV)

104-Rf-259

MAT 459

(n,t) Levels  
293 Kelvin Cross Sections

104-Rf-259



14

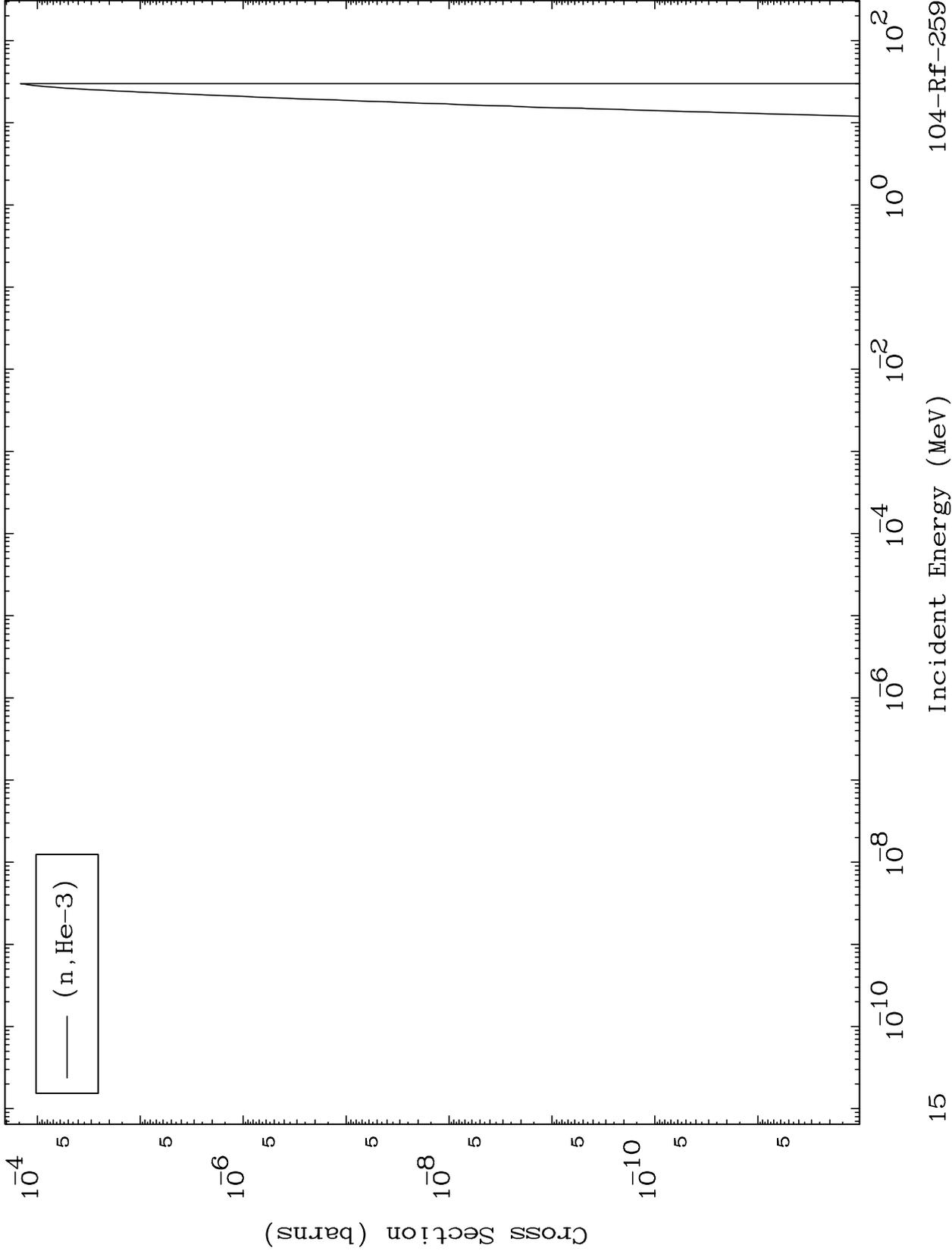
Incident Energy (MeV)

104-Rf-259

MAT 459

(n,He3) Levels  
293 Kelvin Cross Sections

104-Rf-259

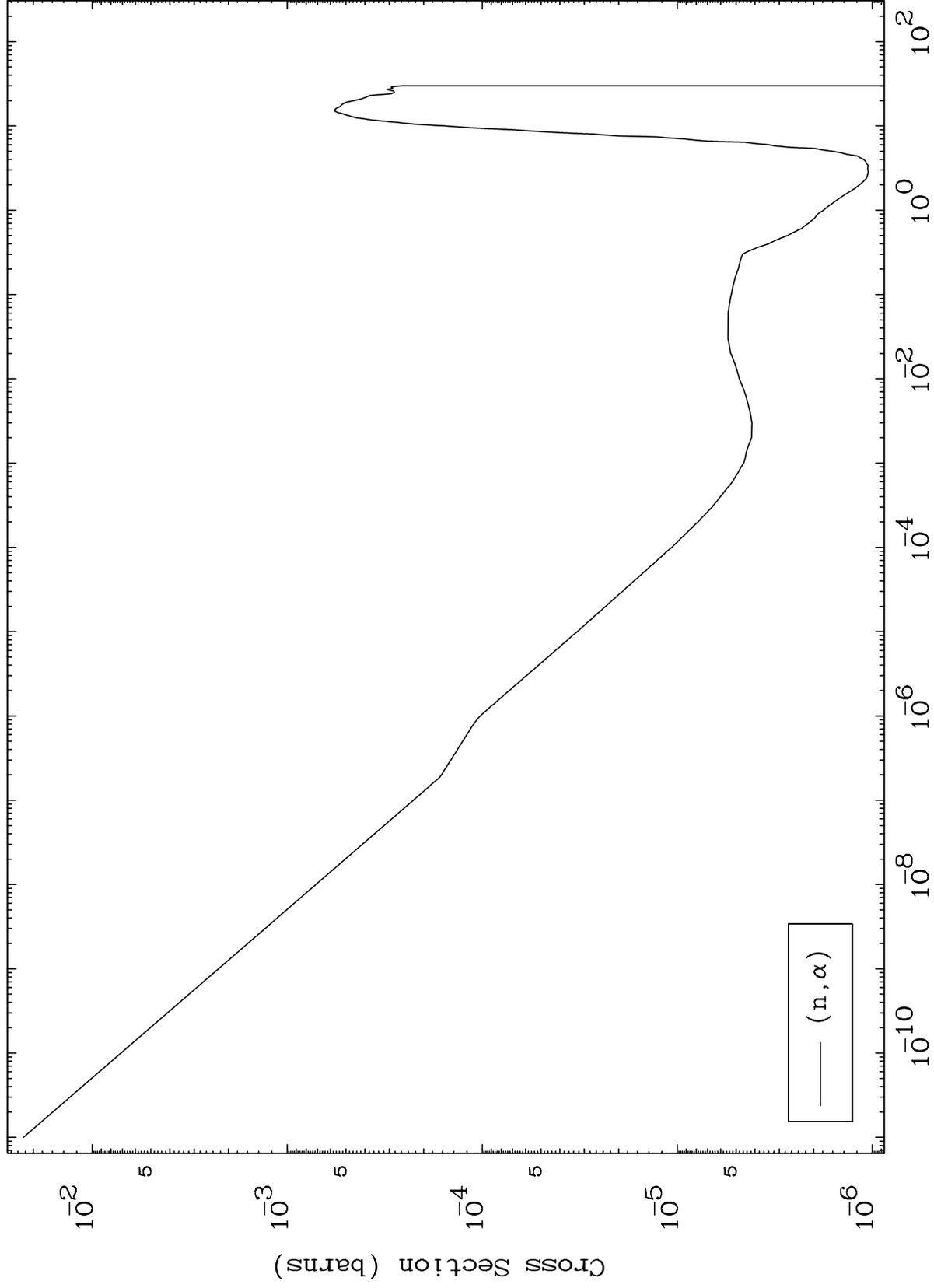


15

MAT 459

(n,α) Levels  
293 Kelvin Cross Sections

104-Rf-259



16

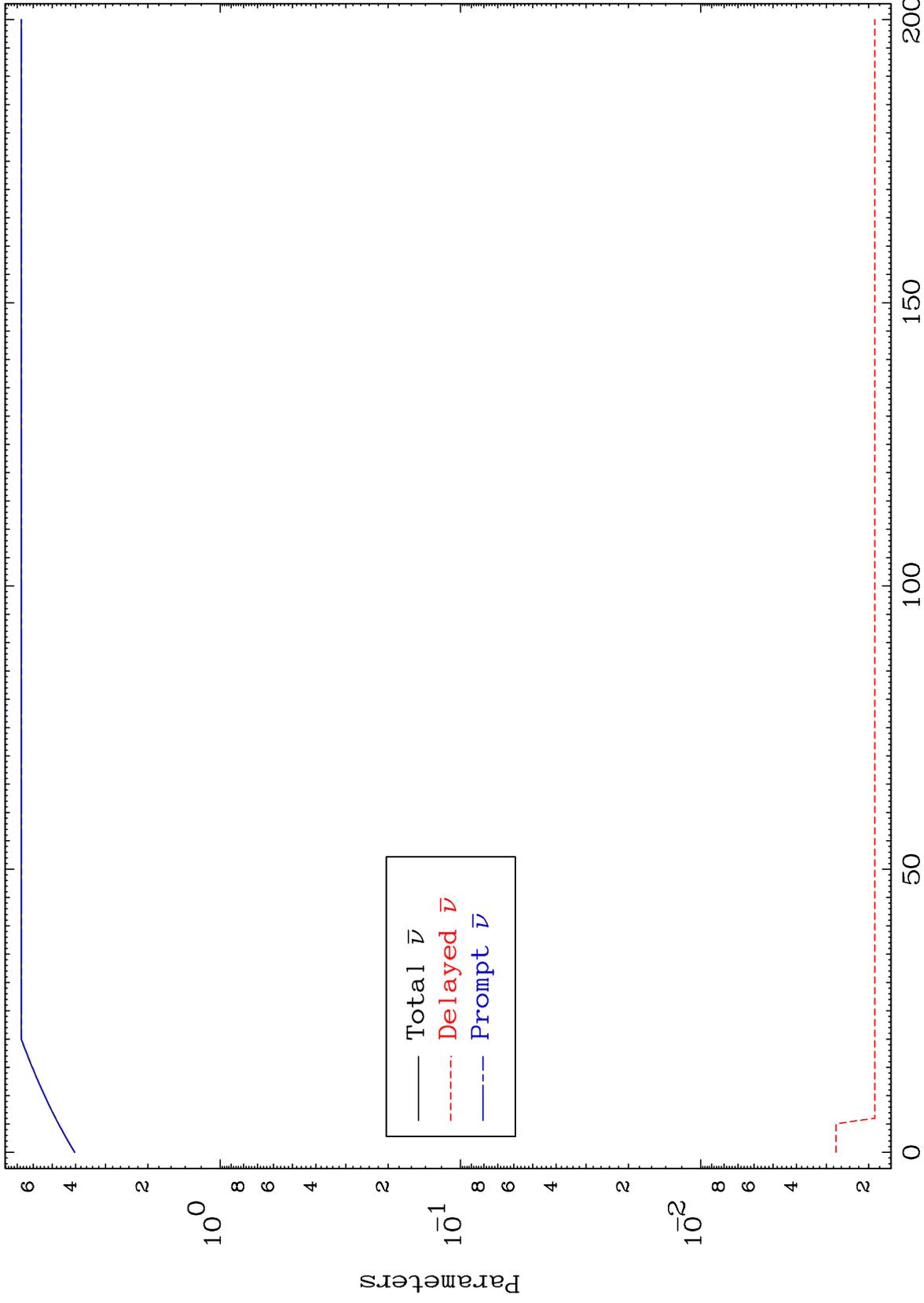
Incident Energy (MeV)

104-Rf-259

MAT 459

Energy Release  
Parameters

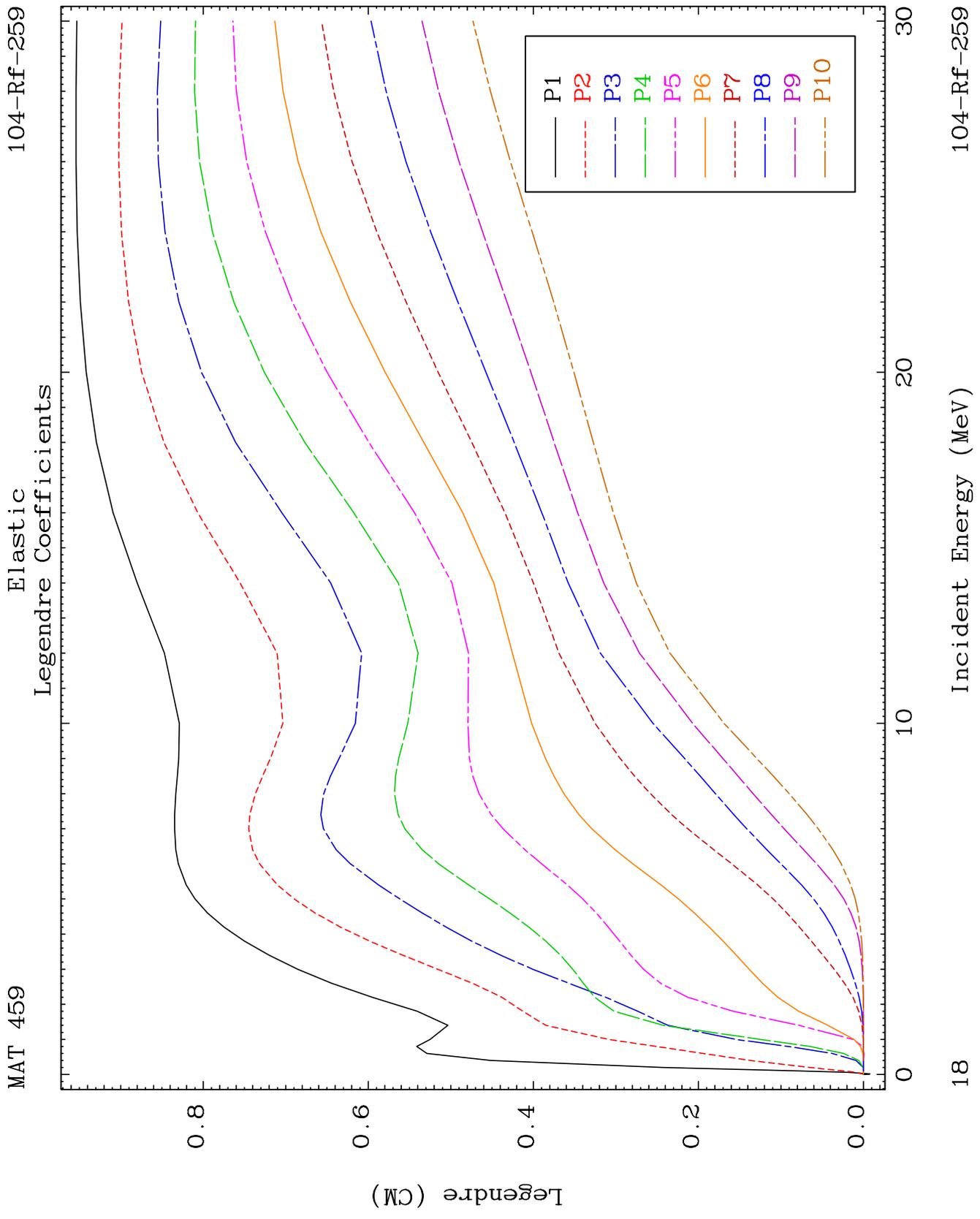
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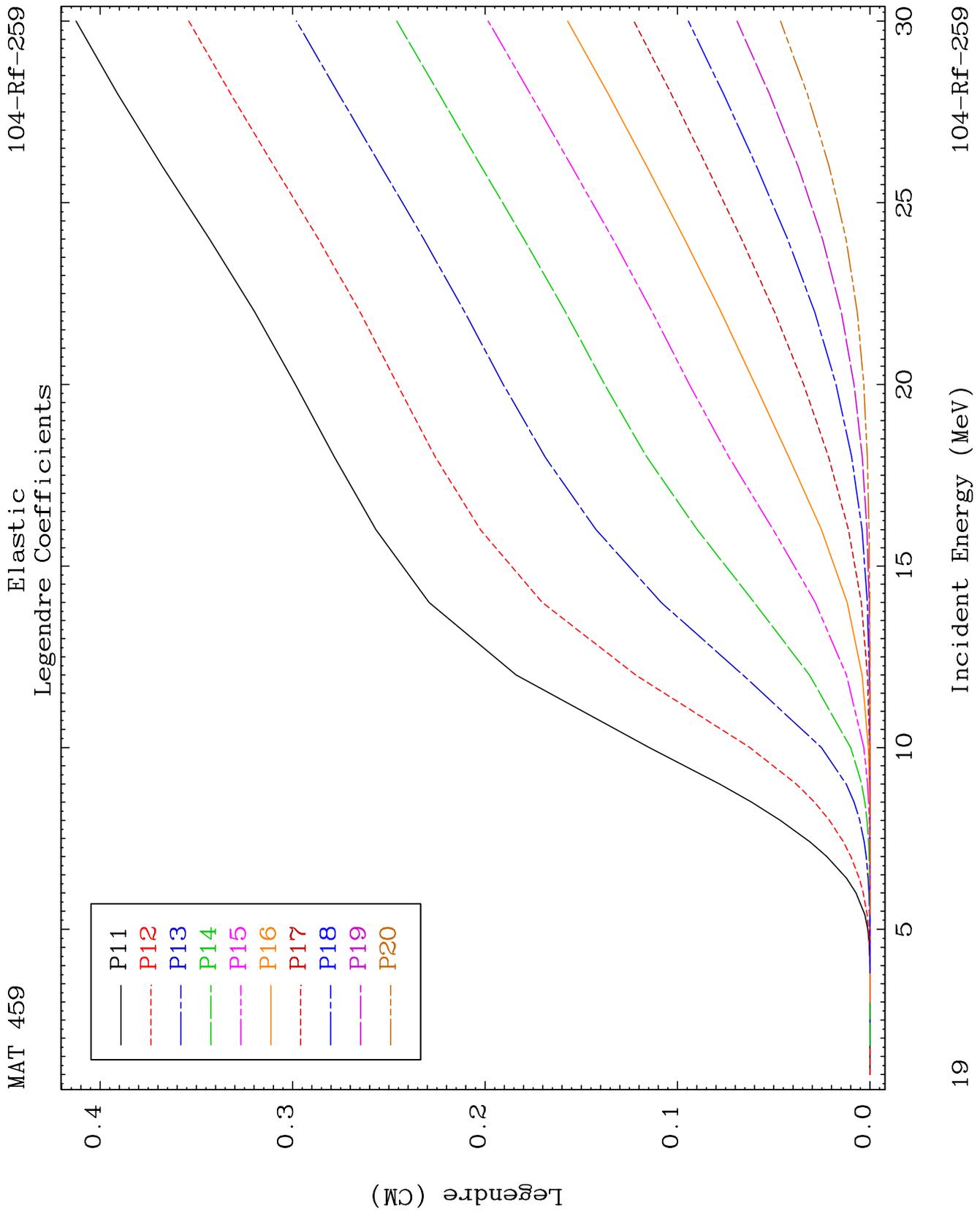


17

Incident Energy (MeV)

104-Rf-259





MAT 459

Elastic Legendre Coefficients

104-Rf-259

19

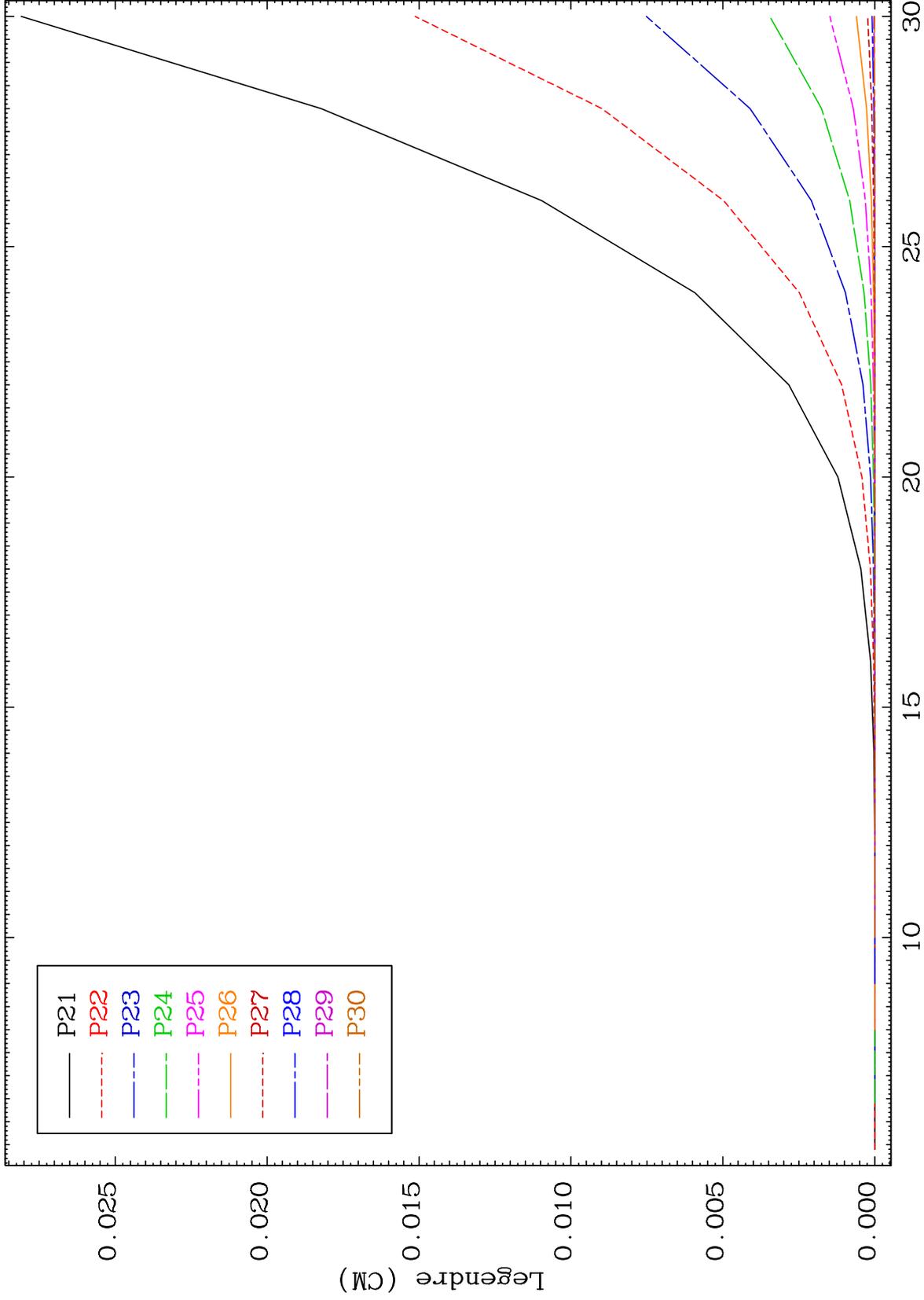
Incident Energy (MeV)

104-Rf-259

MAT 459

### Elastic Legendre Coefficients

104-Rf-259



20

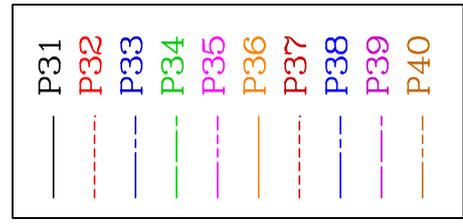
Incident Energy (MeV)

104-Rf-259

MAT 459

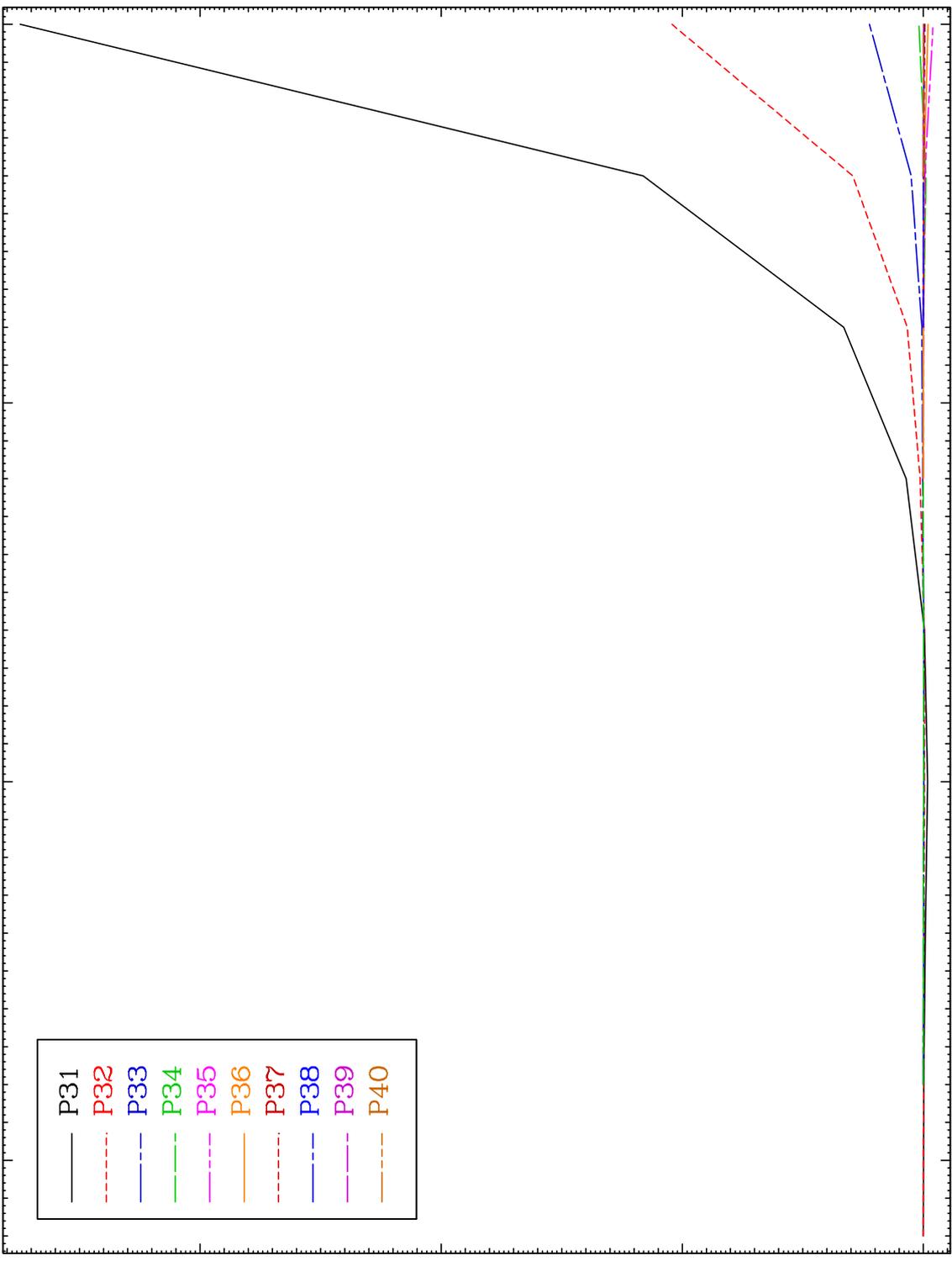
Elastic Legendre Coefficients

104-Rf-259



$\times 10^{-6}$

Legendre (CM)



30

25

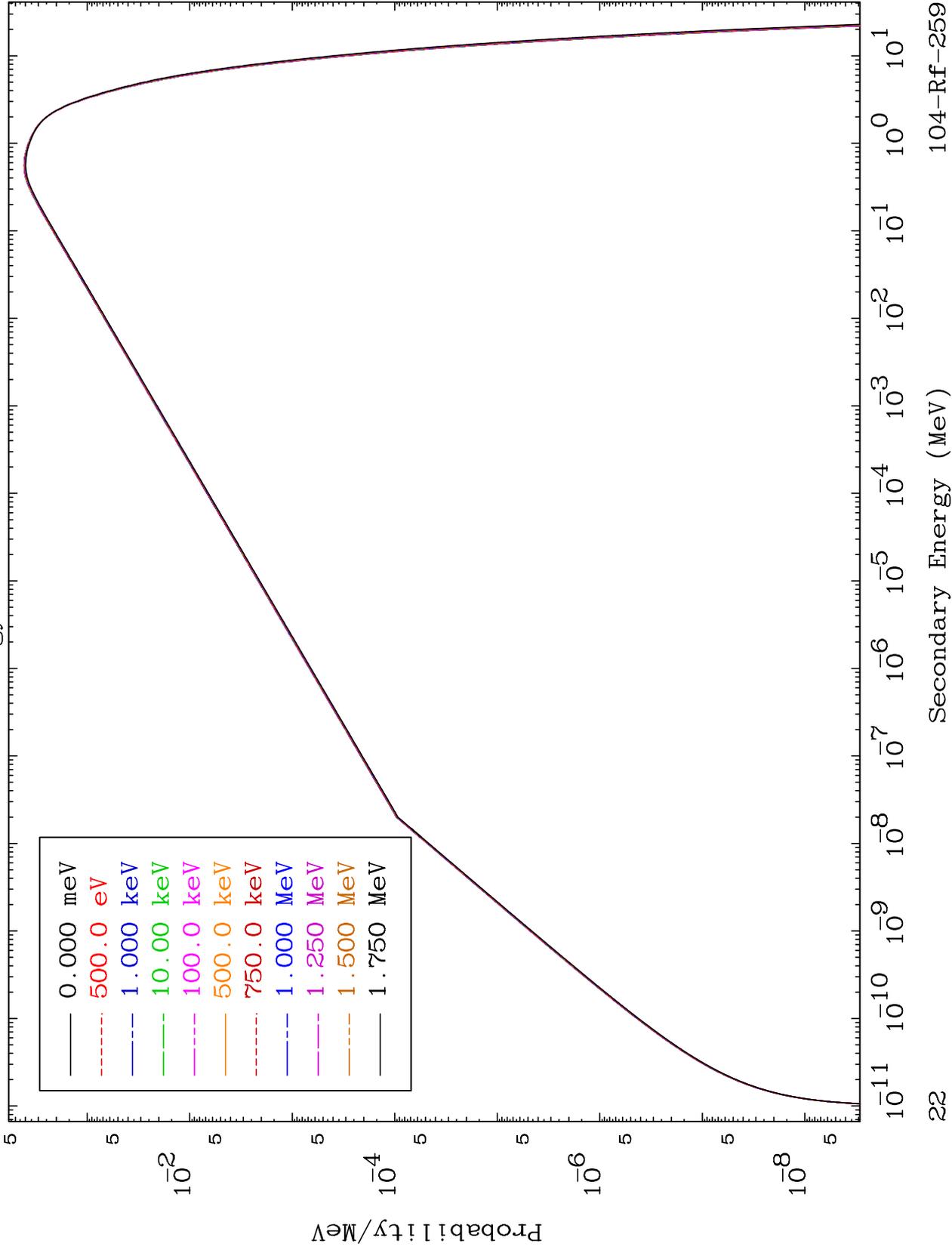
20

15

21

Incident Energy (MeV)

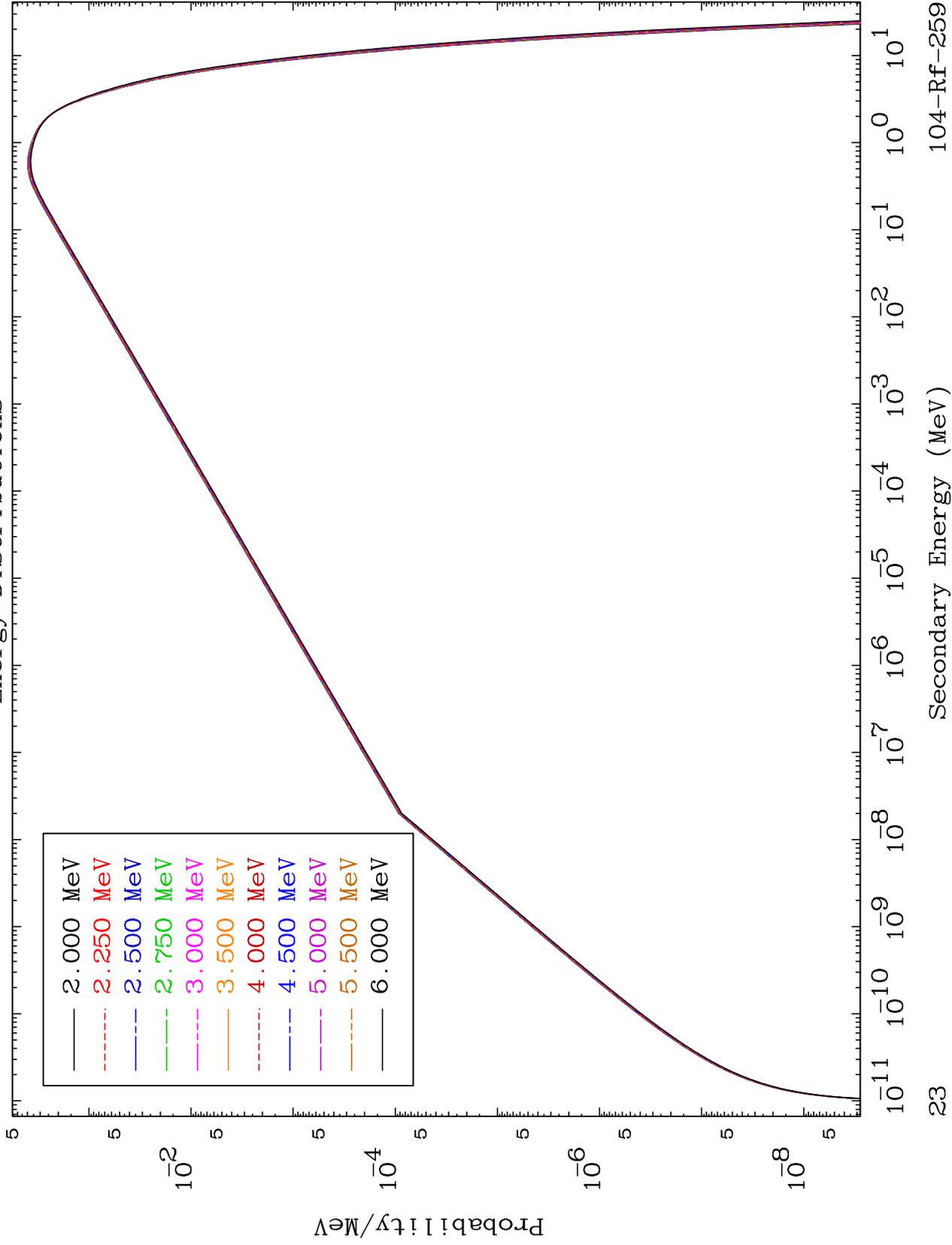
104-Rf-259



MAT 459

### Fission Energy Distributions

104-Rf-259



104-Rf-259

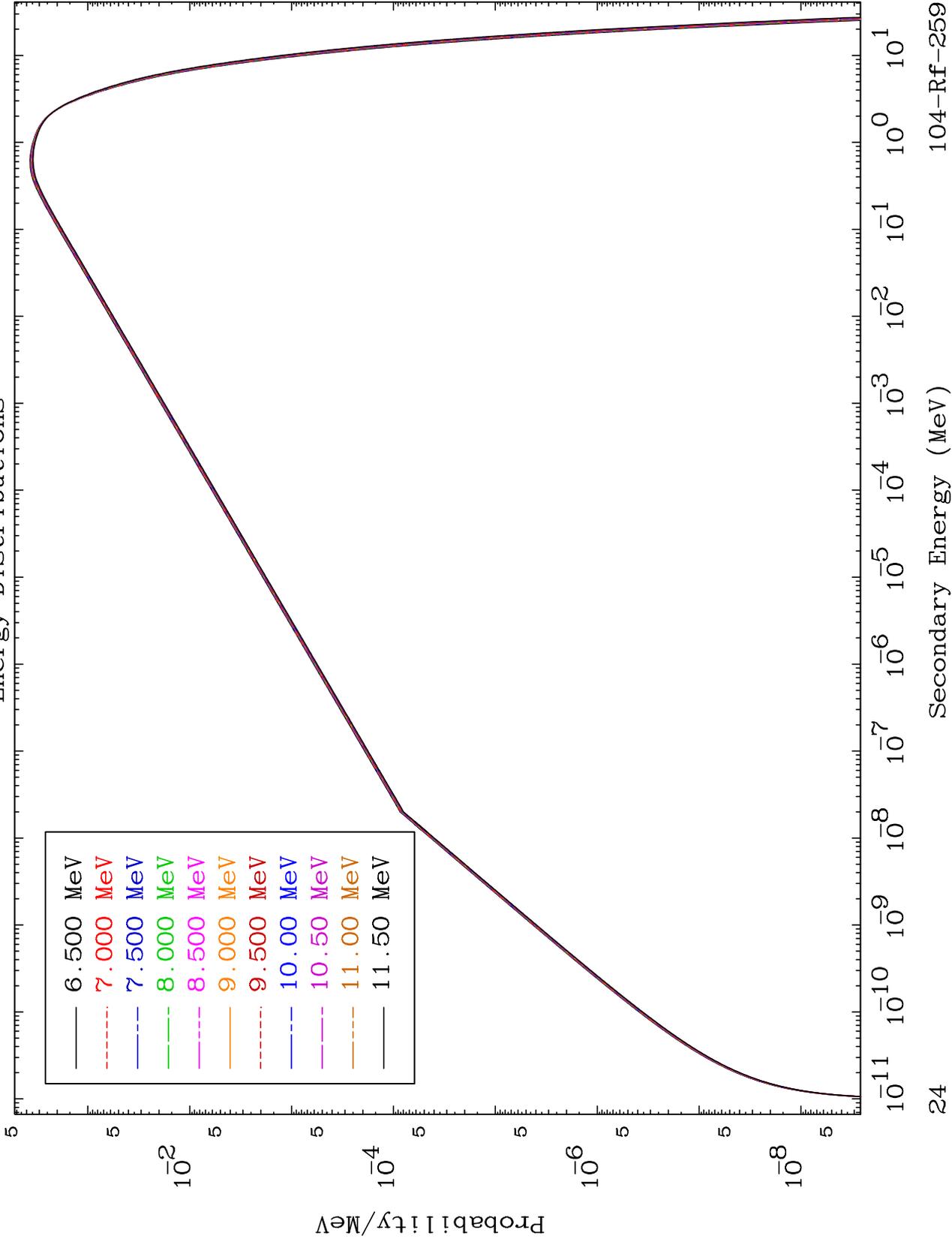
Secondary Energy (MeV)

23

MAT 459

### Fission Energy Distributions

104-Rf-259



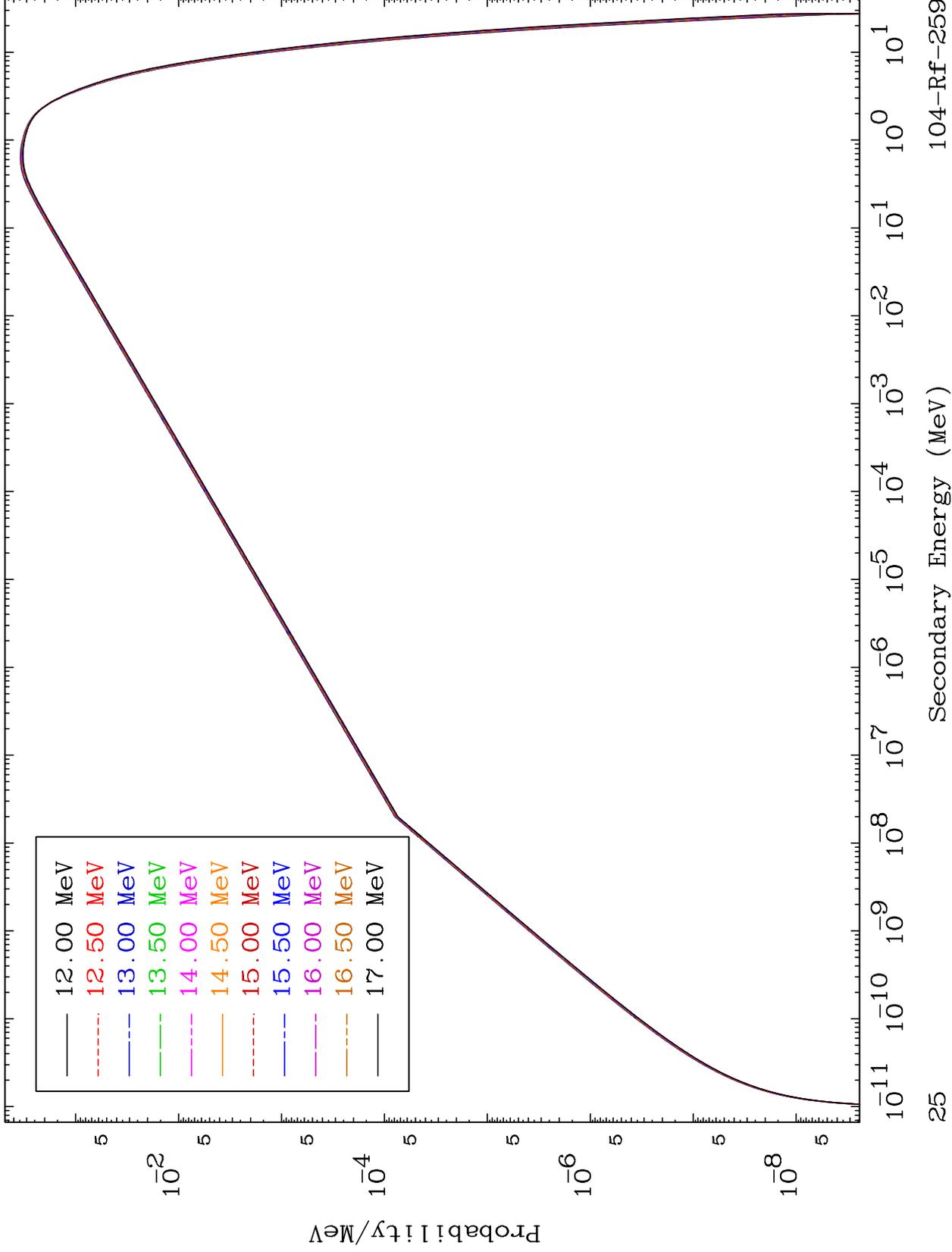
24

104-Rf-259

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### Fission Energy Distributions

104-Rf-259



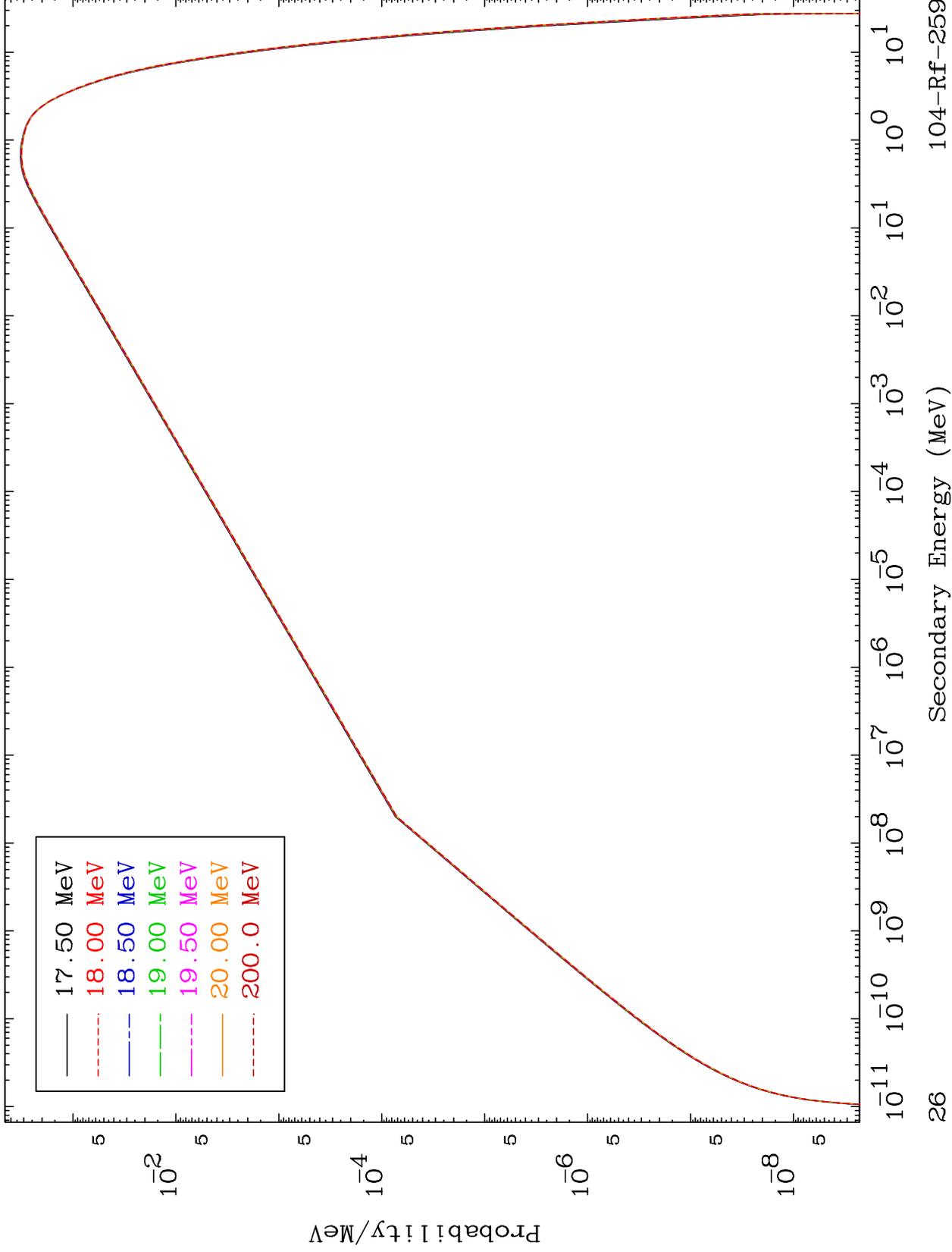
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104-Rf-259

MAT 459

### Fission Energy Distributions

104-Rf-259



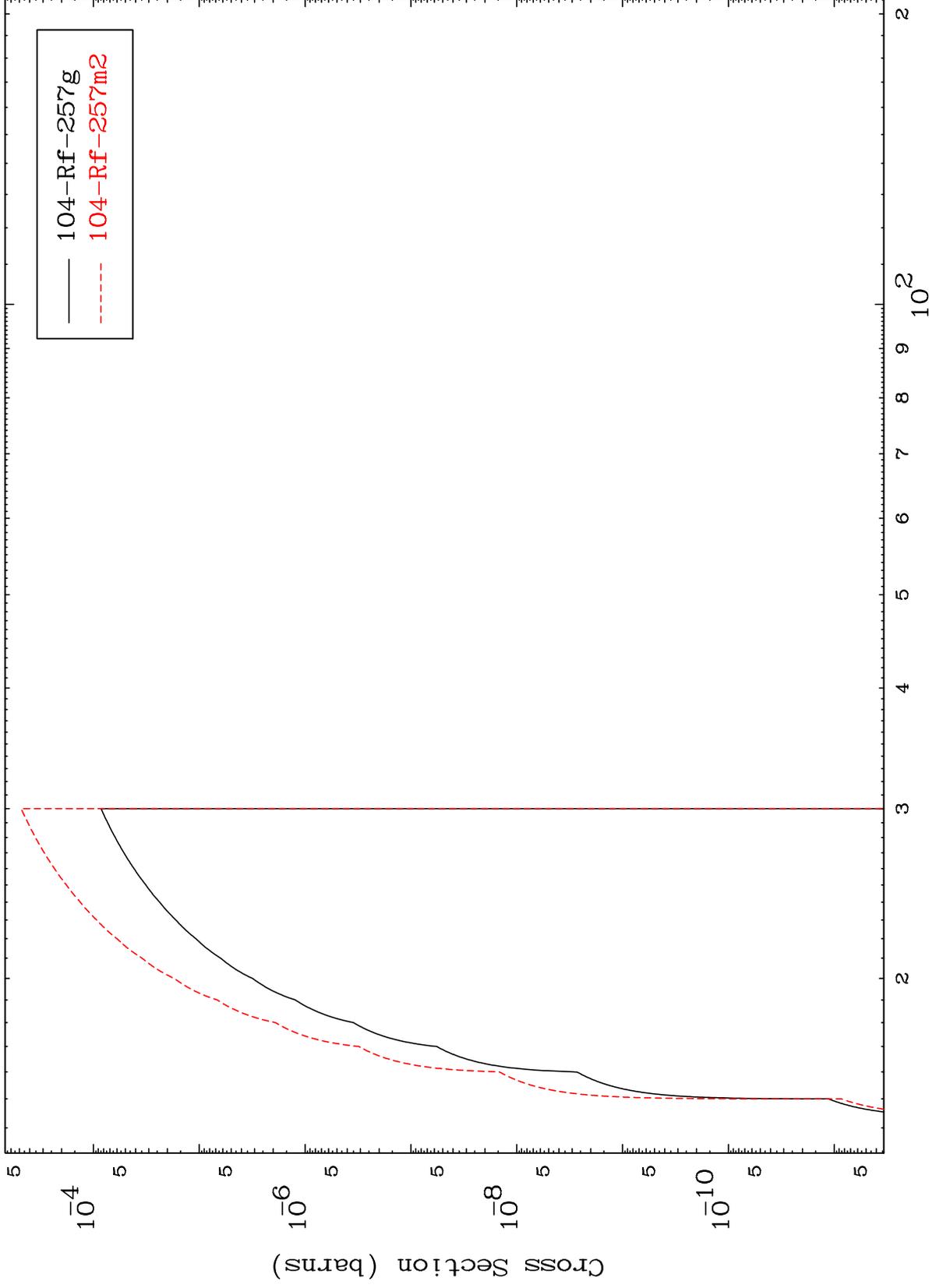
26

MAT 459

(n,3n)

104-Rf-259

Radionuclide Production Cross Section



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Incident Energy (MeV)

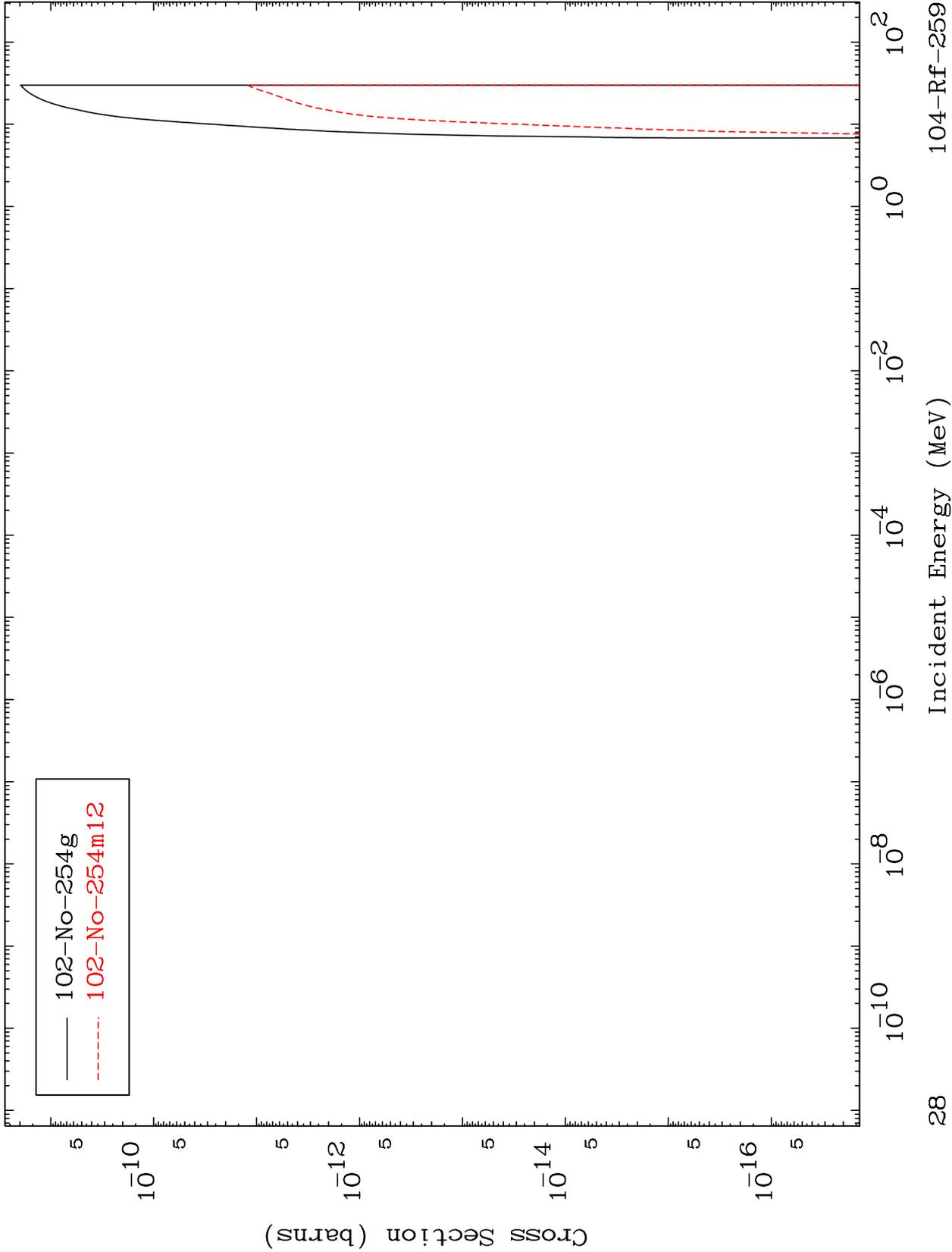
104-Rf-259

MAT 459

(n,2n)  $\alpha$

104-Rf-259

Radionuclide Production Cross Section

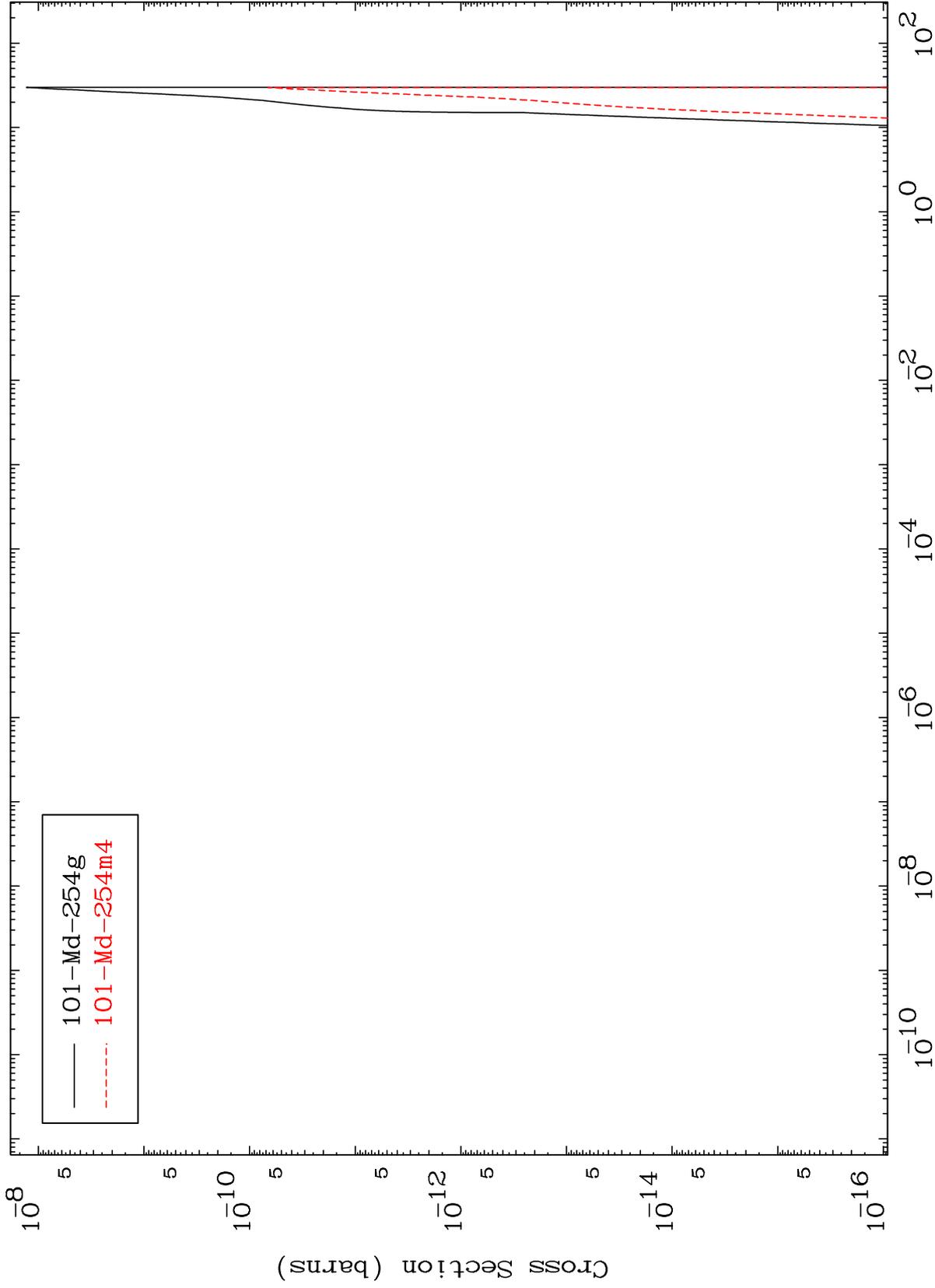


MAT 459

(n,n') p  $\alpha$

104-Rf-259

Radionuclide Production Cross Section



29

Incident Energy (MeV)

104-Rf-259