

Program EVALPLOT
(Version 2021-1)

by

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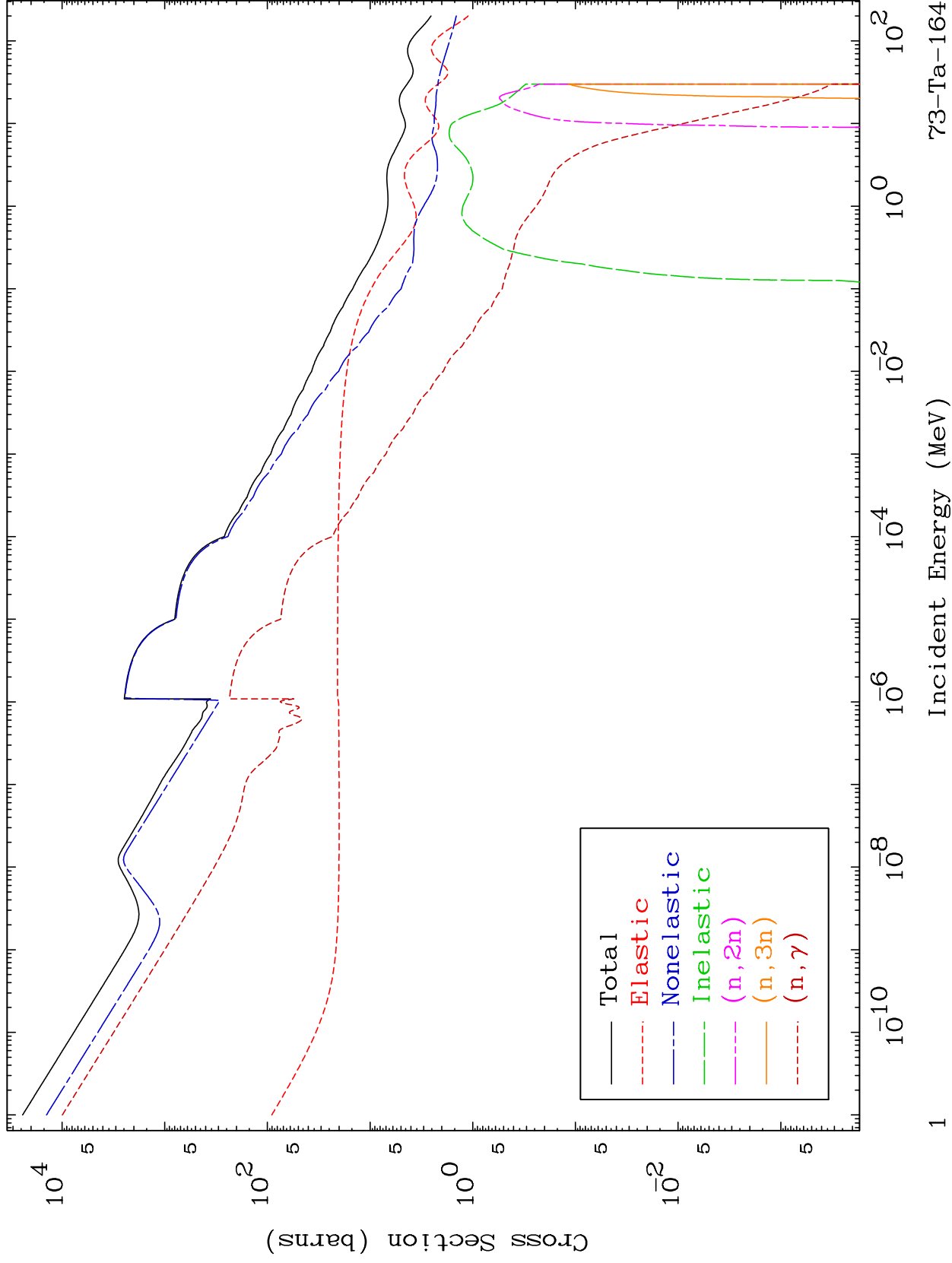
Press Mouse Button to Start

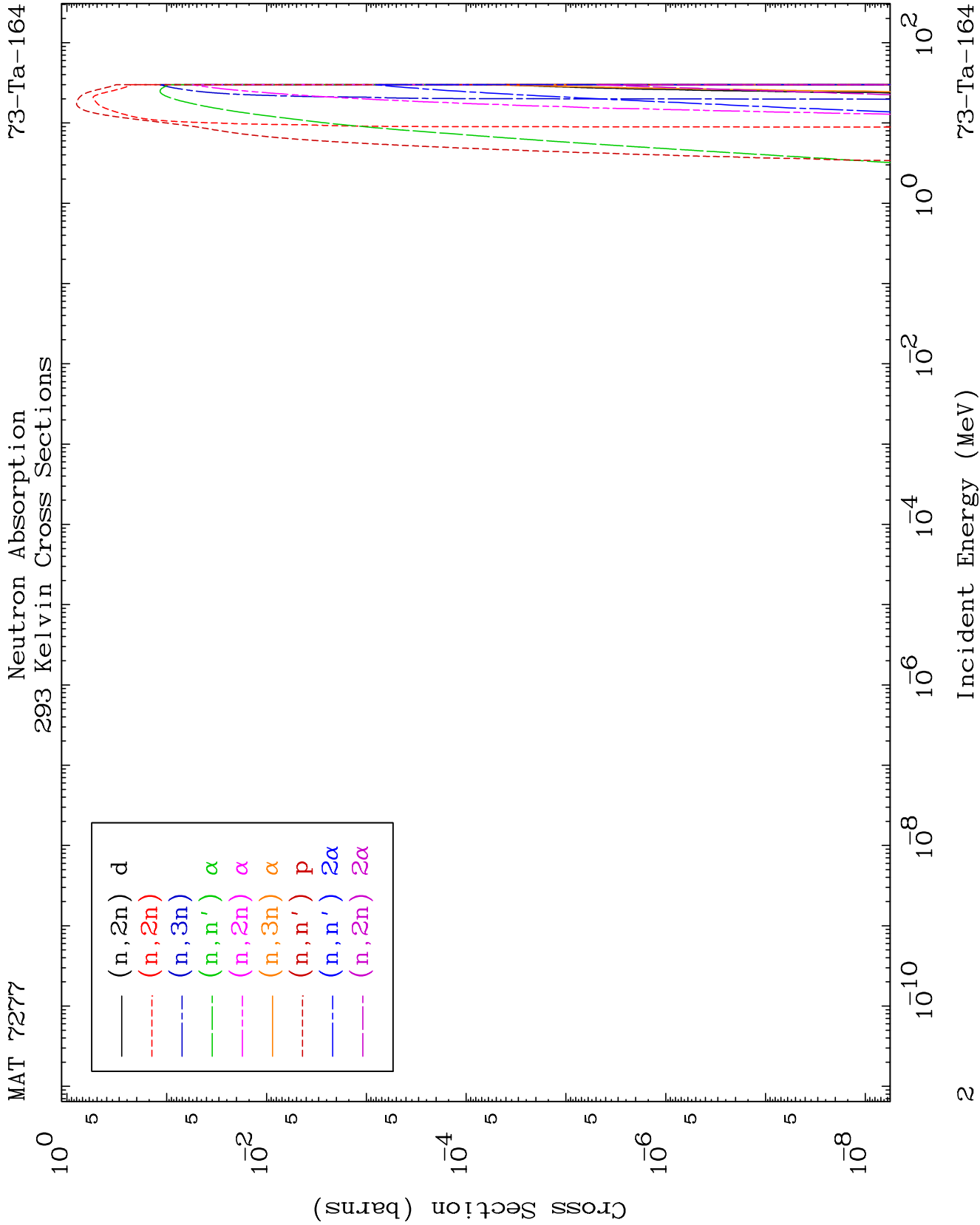
MAT 7277

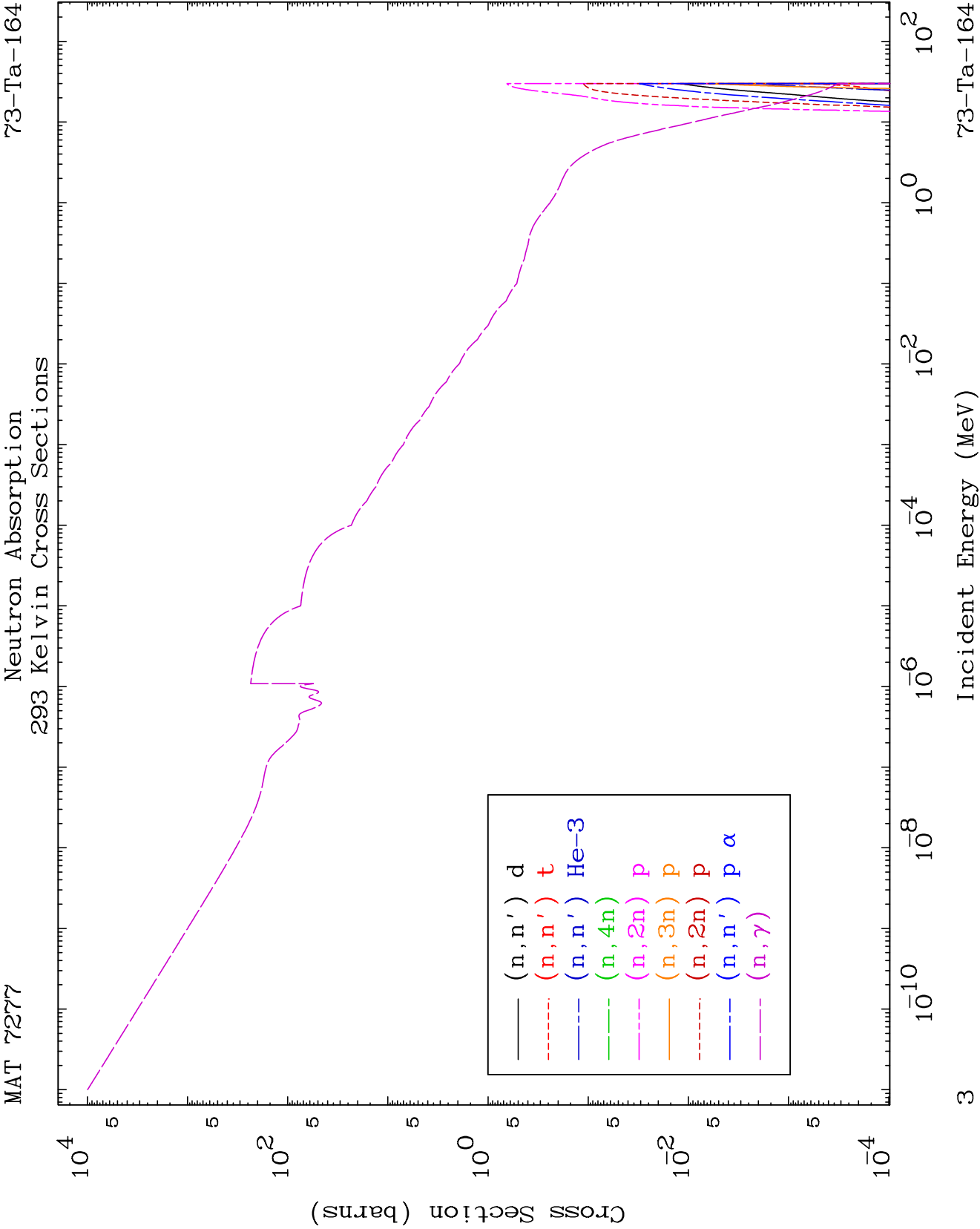
Neutron Major

293 Kelvin Cross Sections

73-Ta-164



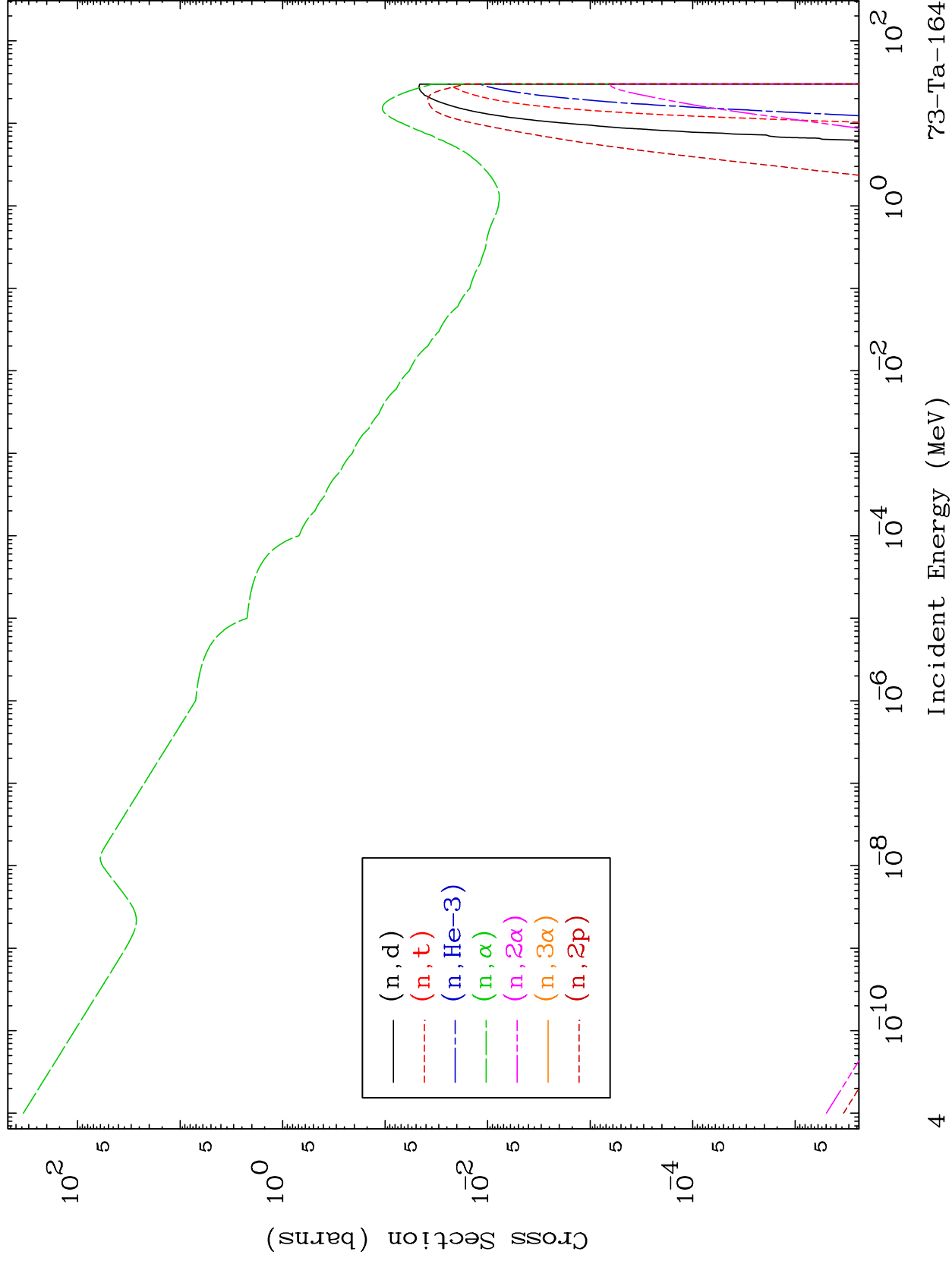


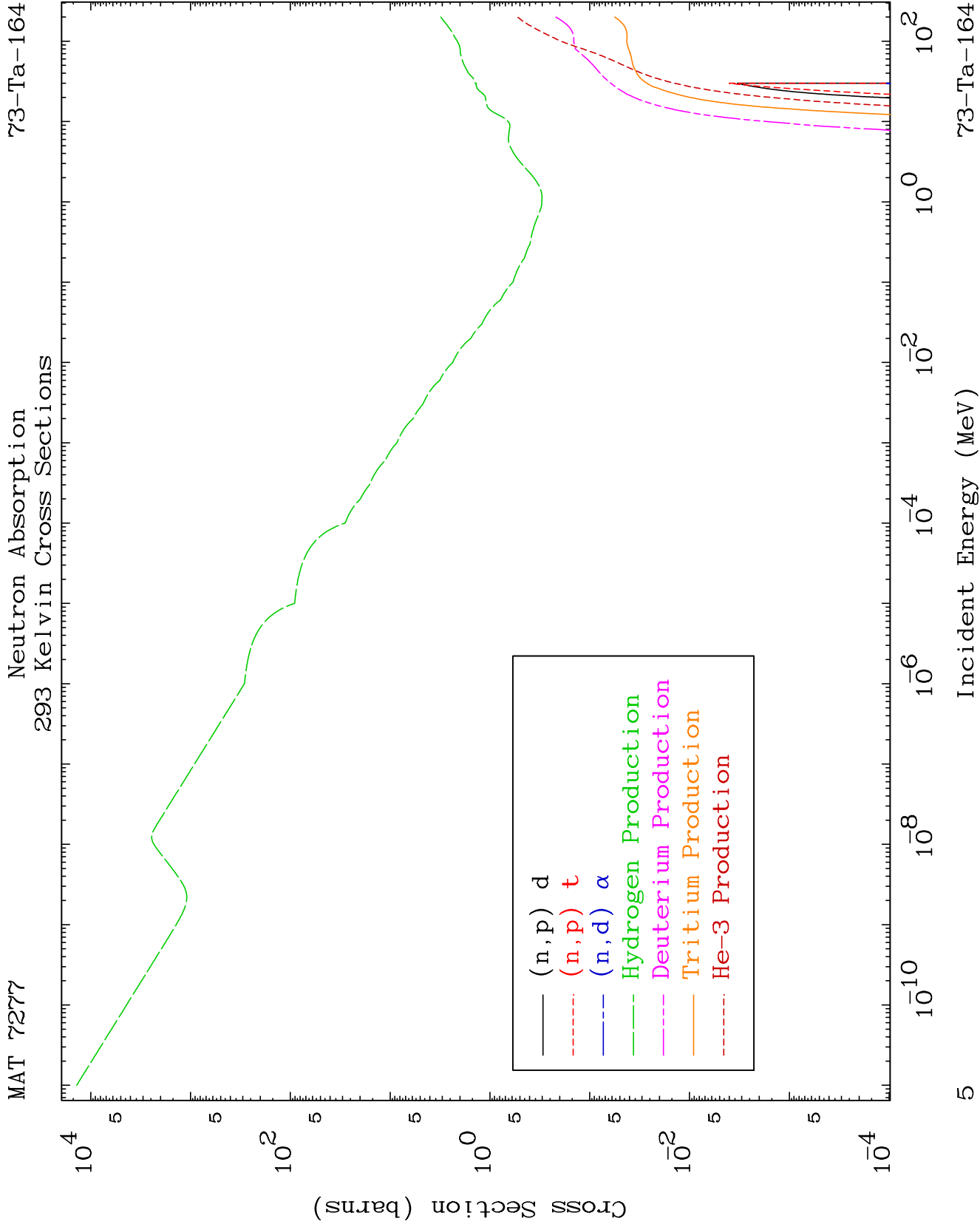


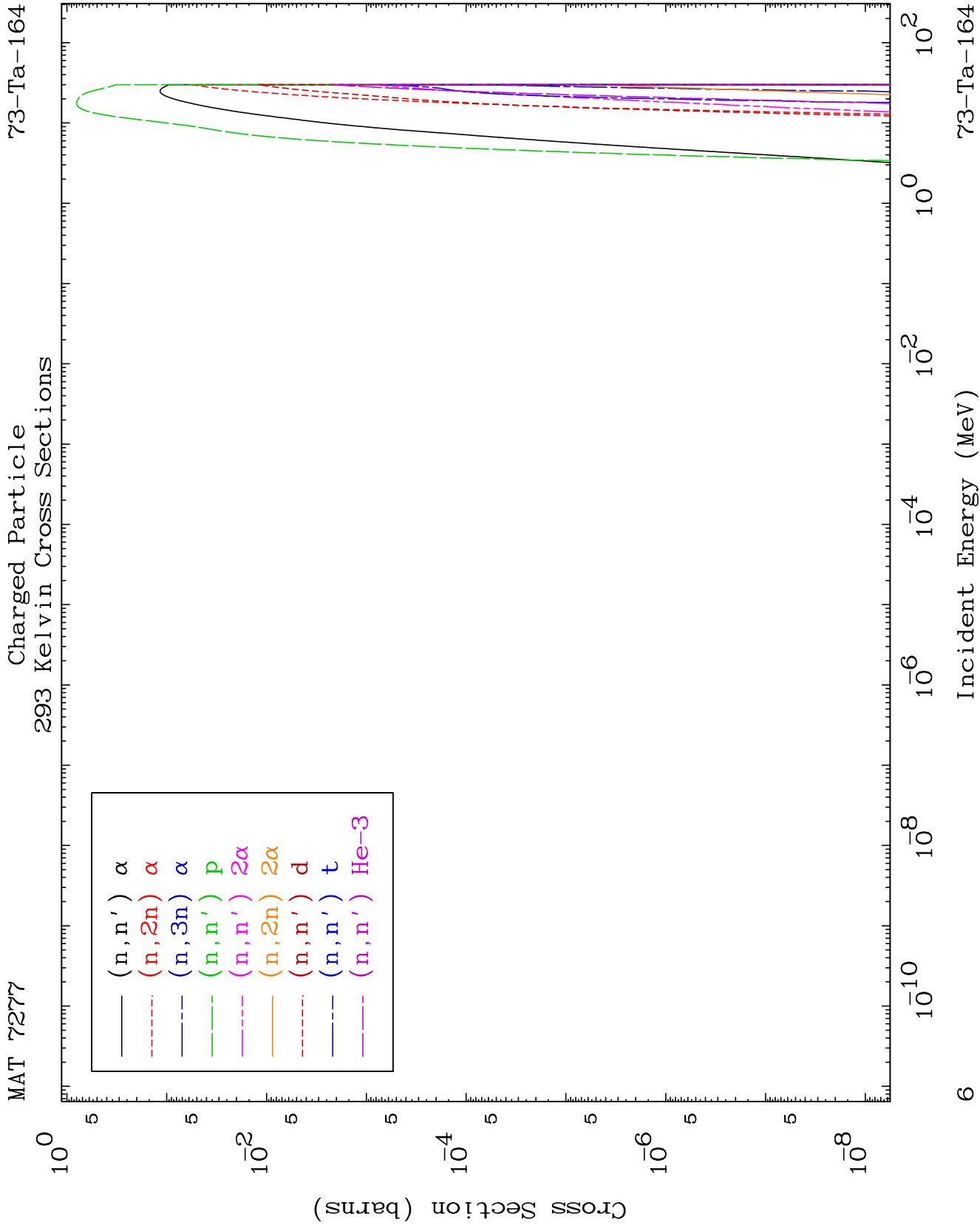
MAT 7277

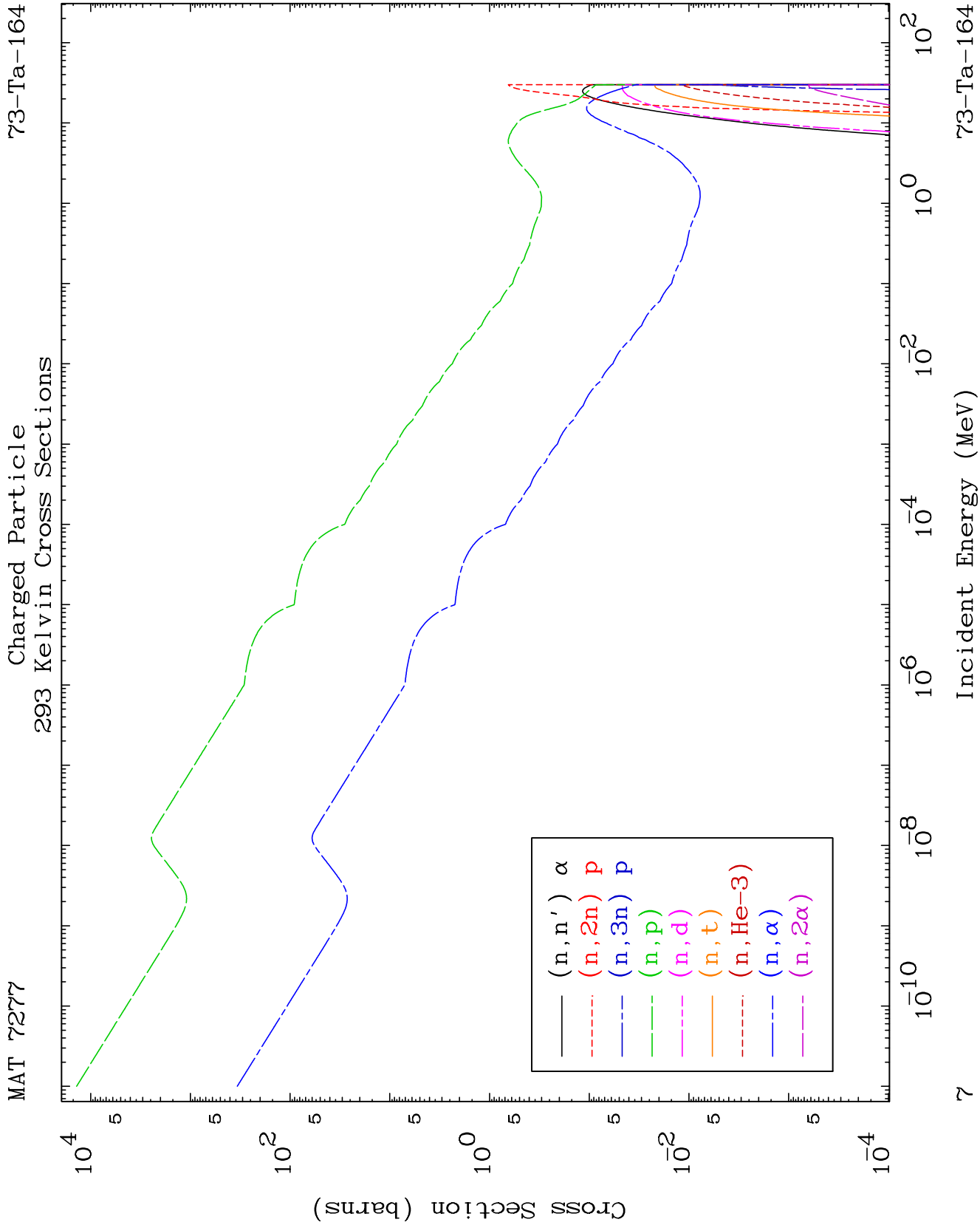
Neutron Absorption
293 Kelvin Cross Sections

73-Ta-164





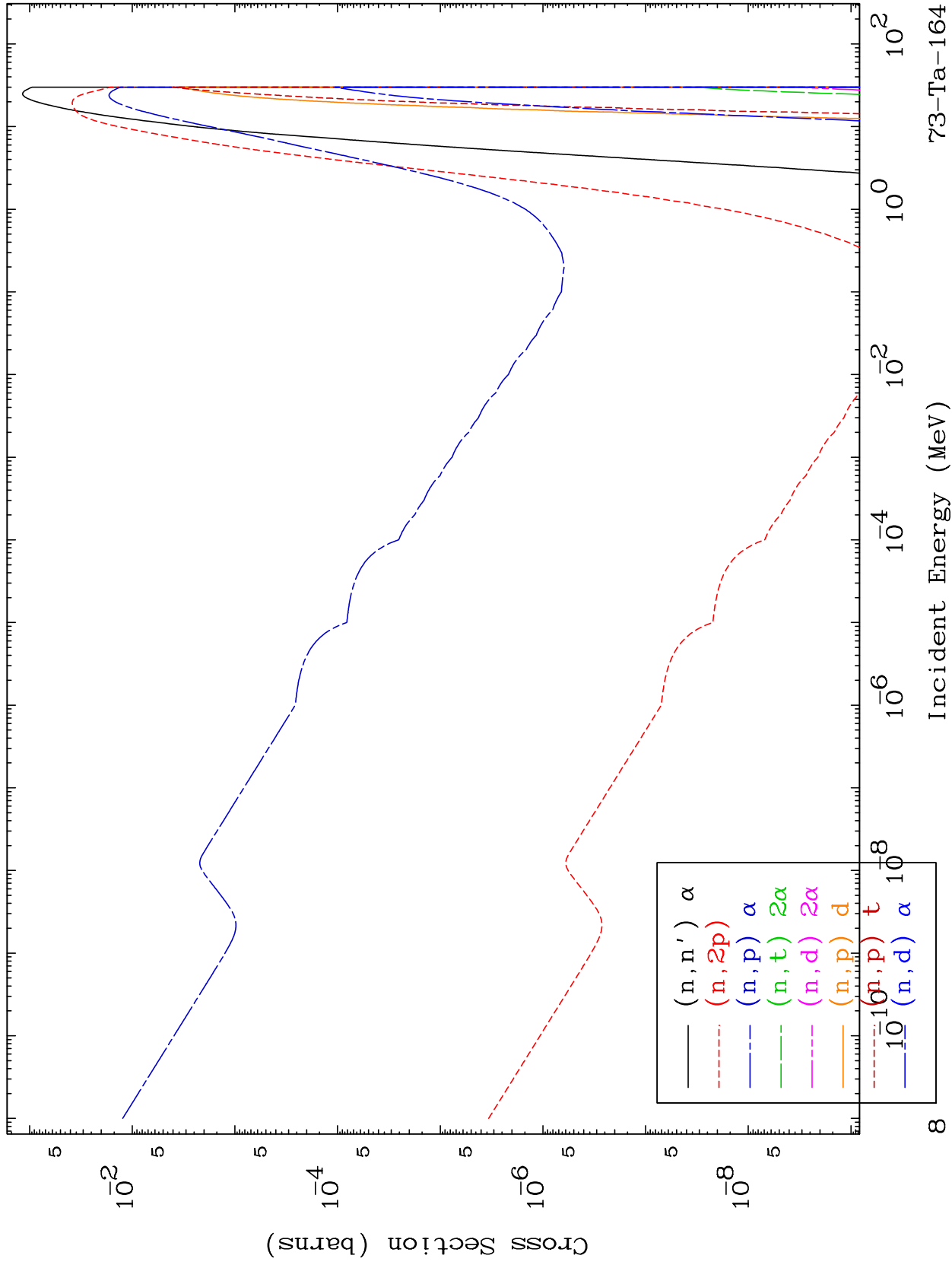


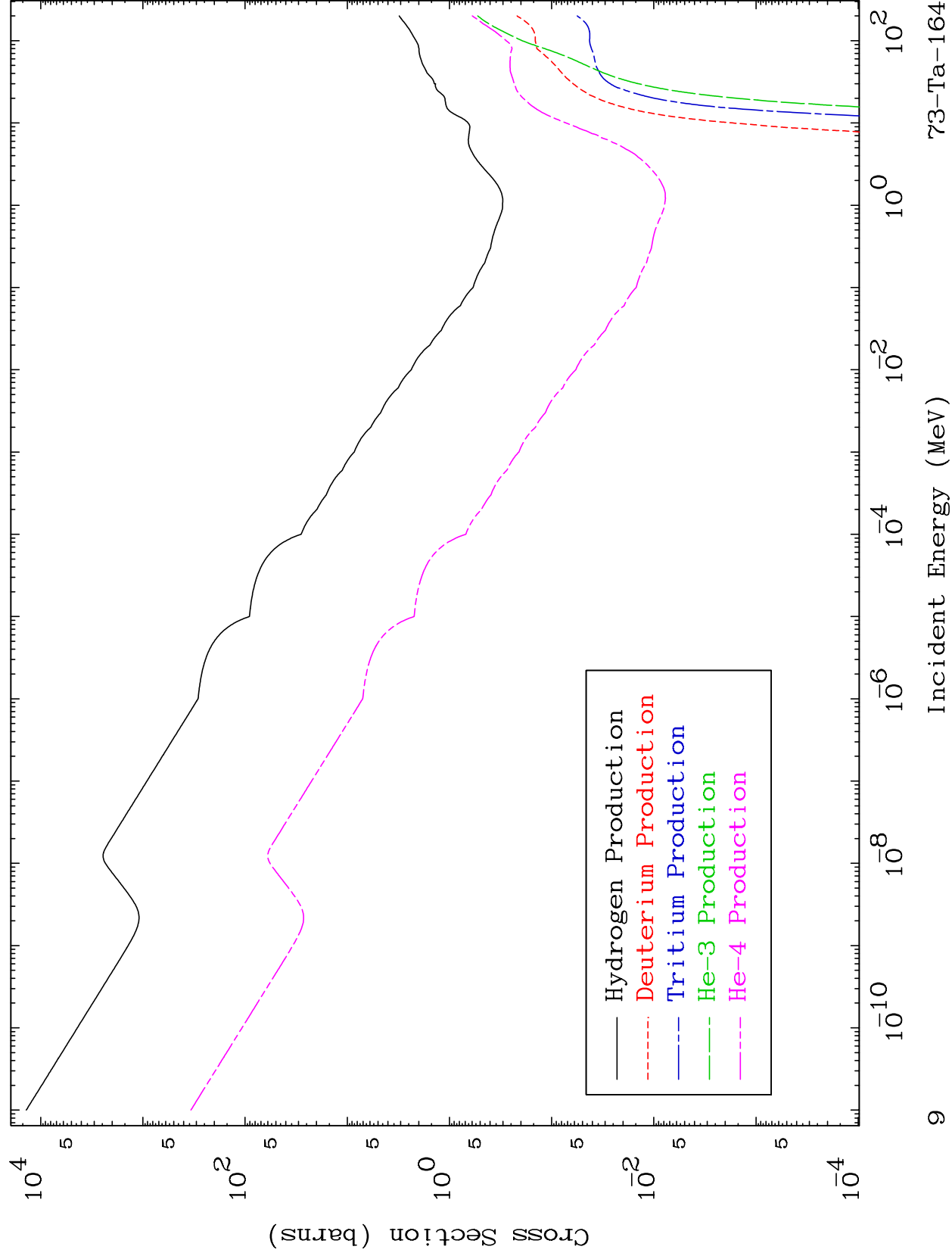


MAT 7277

Charged Particle
293 Kelvin Cross Sections

73-Ta-164

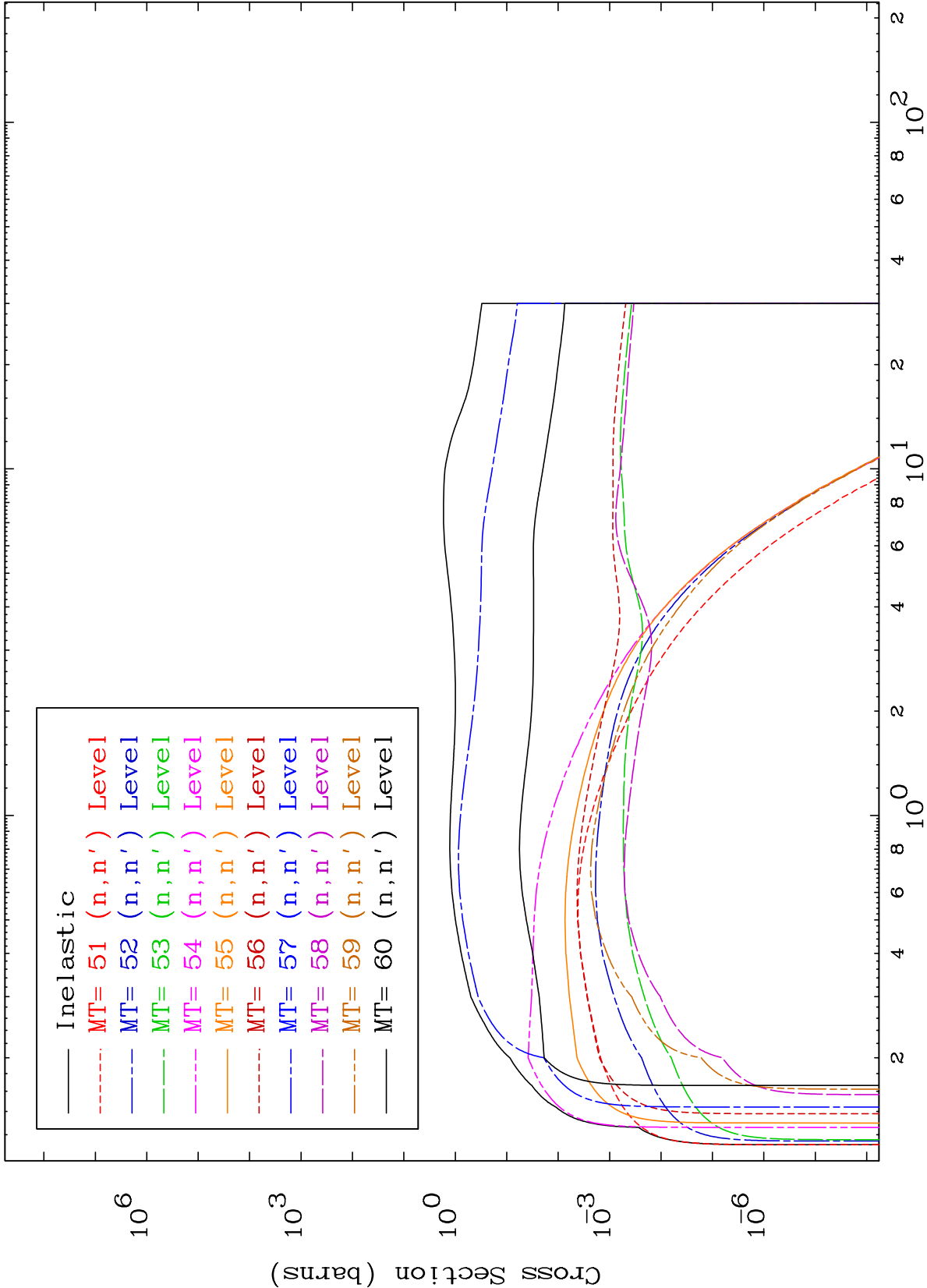




MAT 7277

(n,n') Levels
293 Kelvin Cross Sections

73-Ta-164

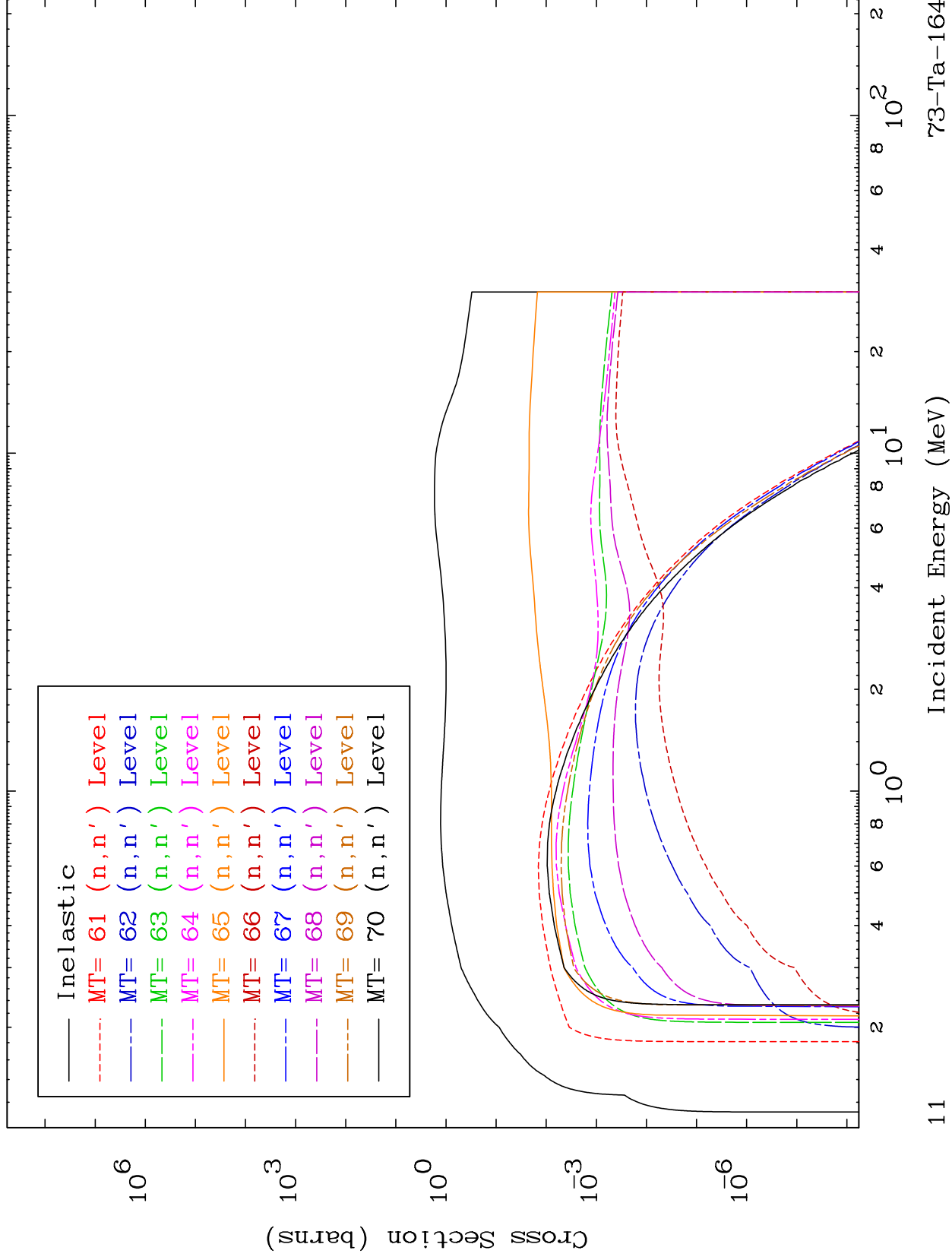


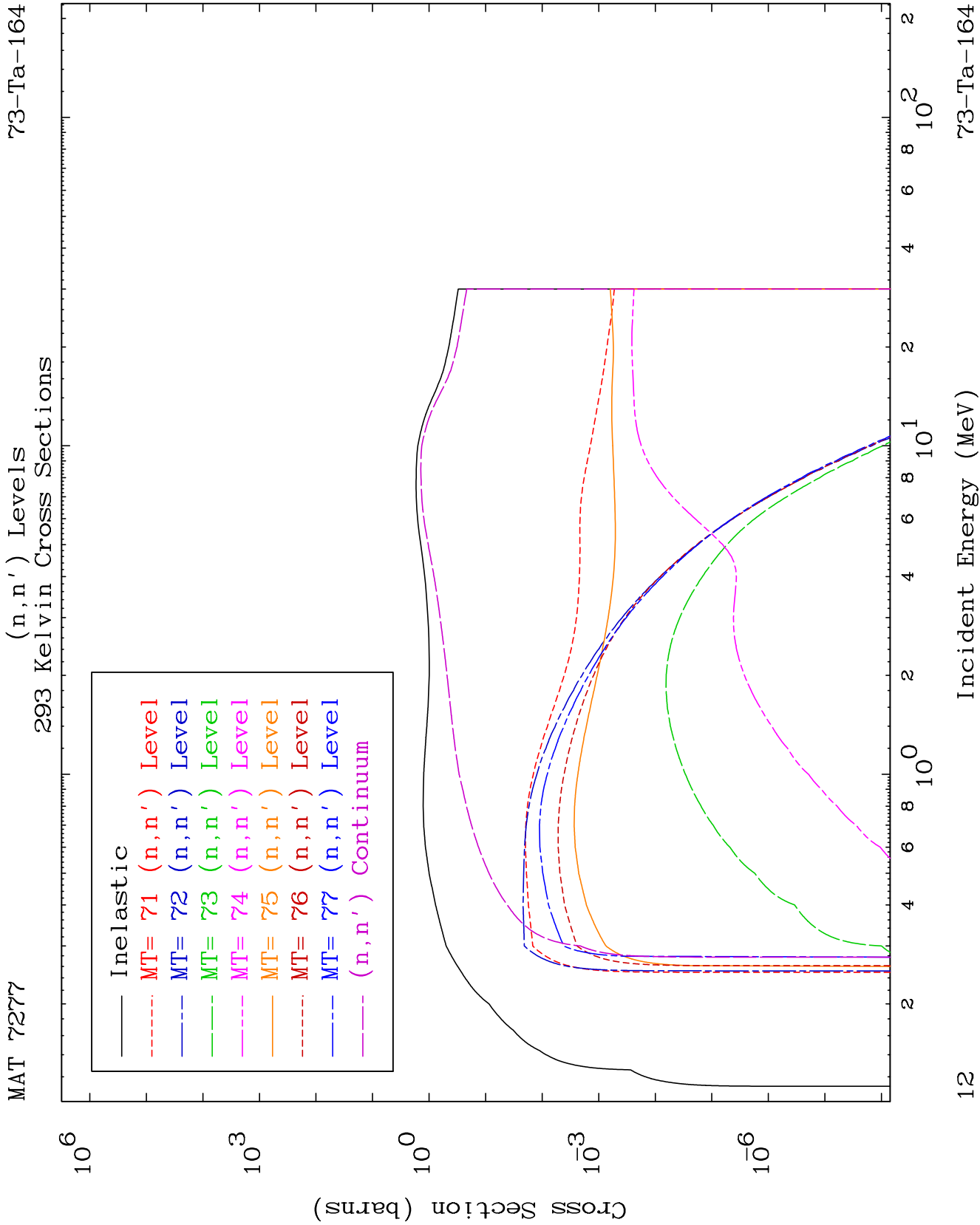
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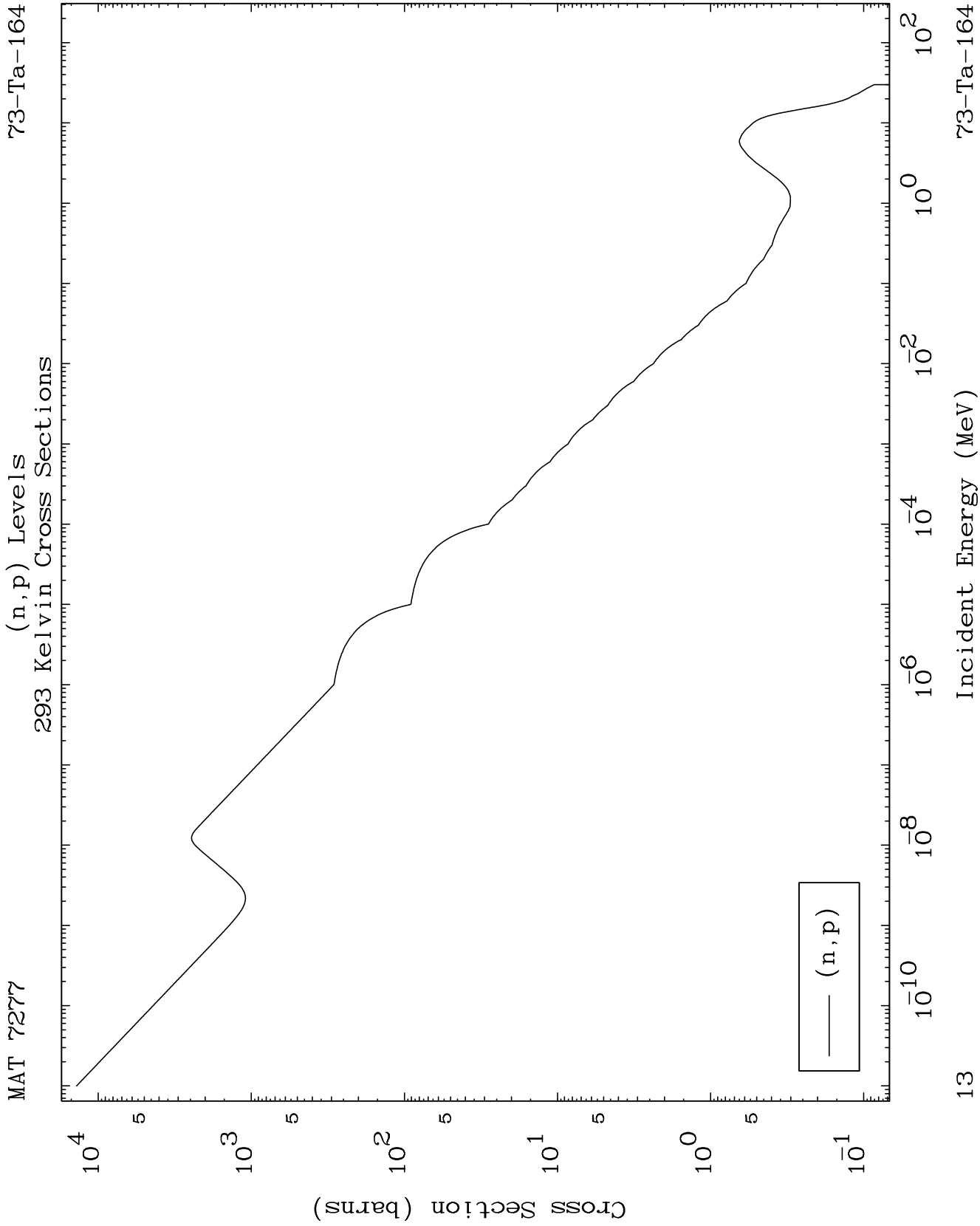
Incident Energy (MeV)

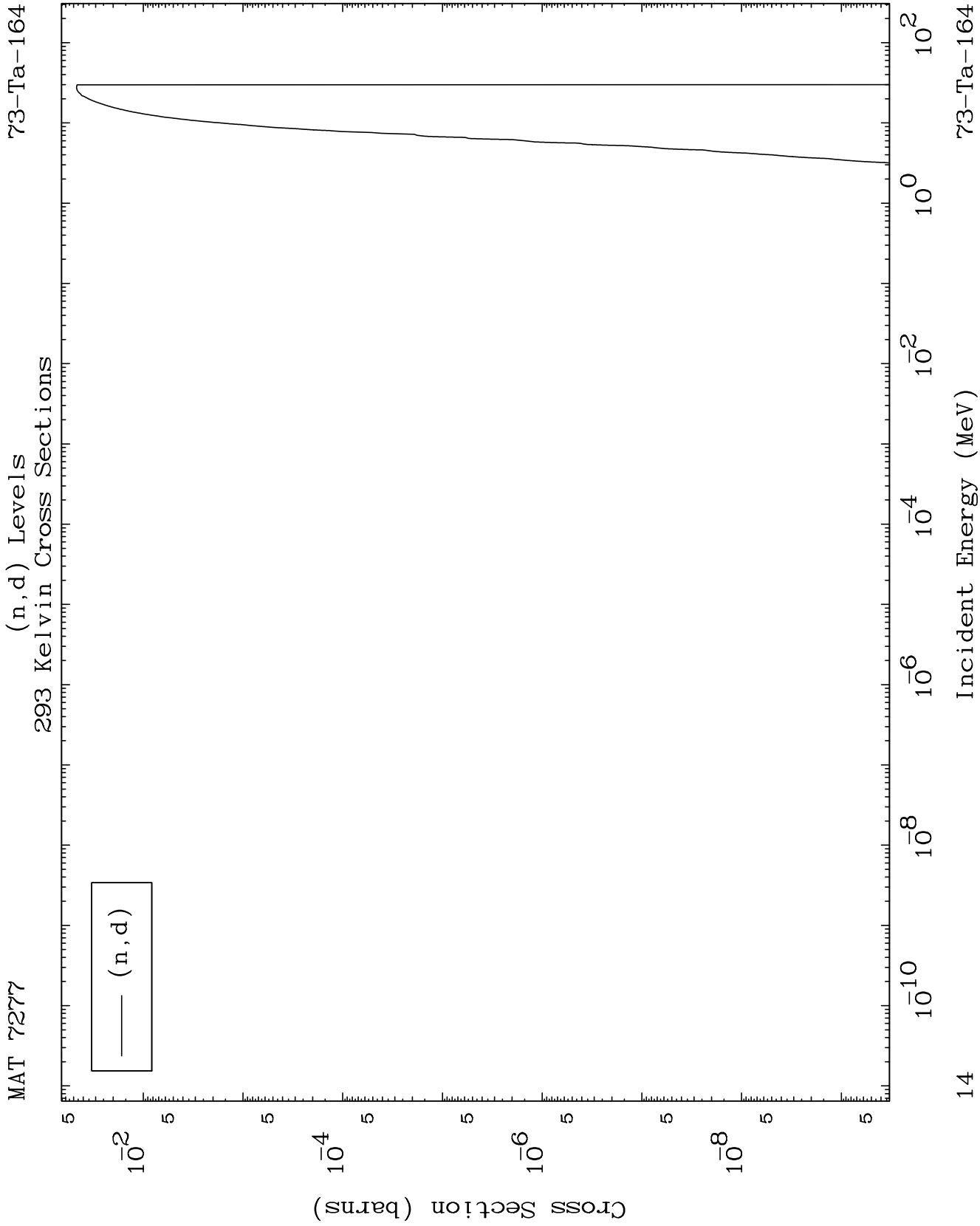
73-Ta-164

(n,n') Levels
293 Kelvin Cross Sections





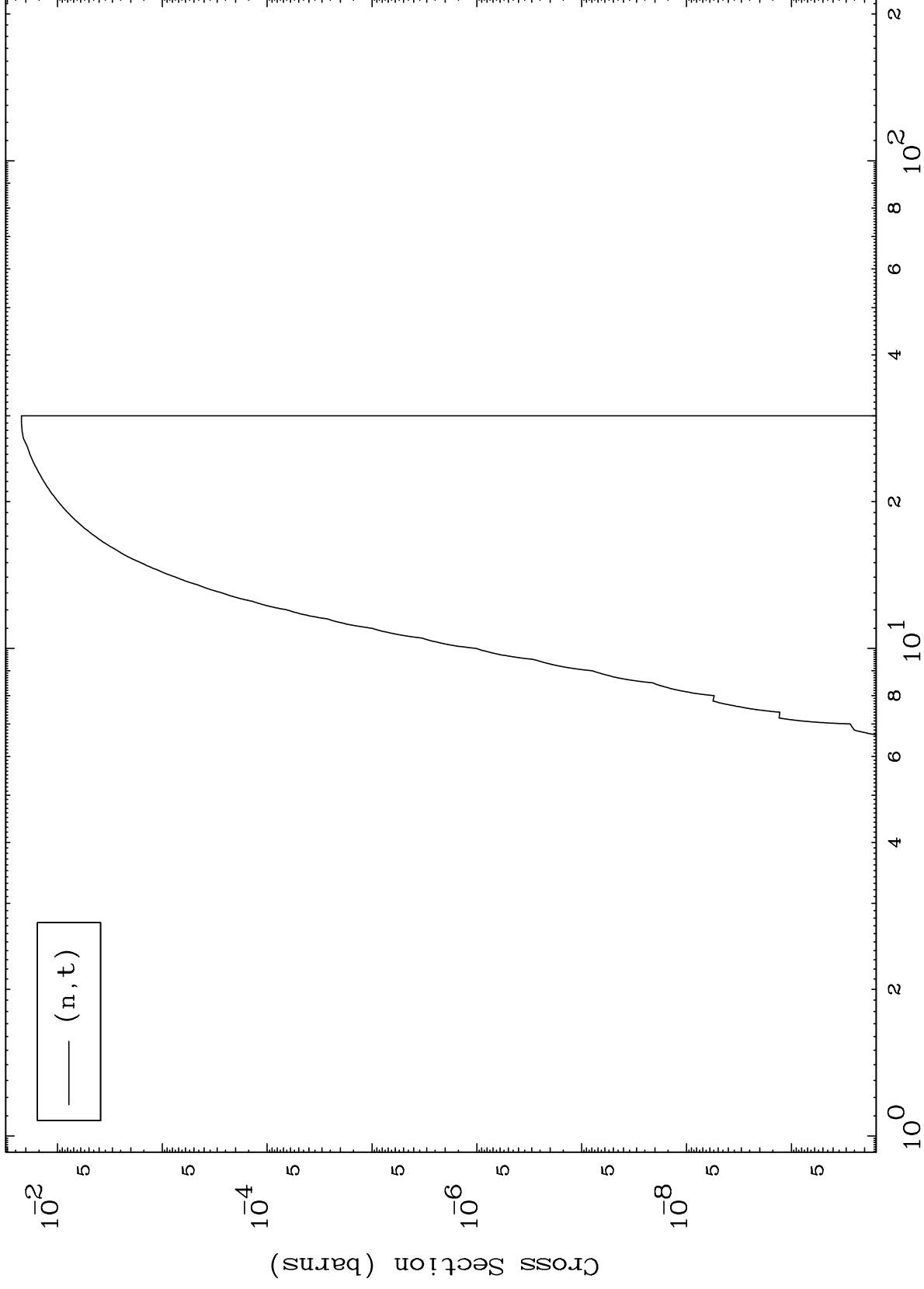




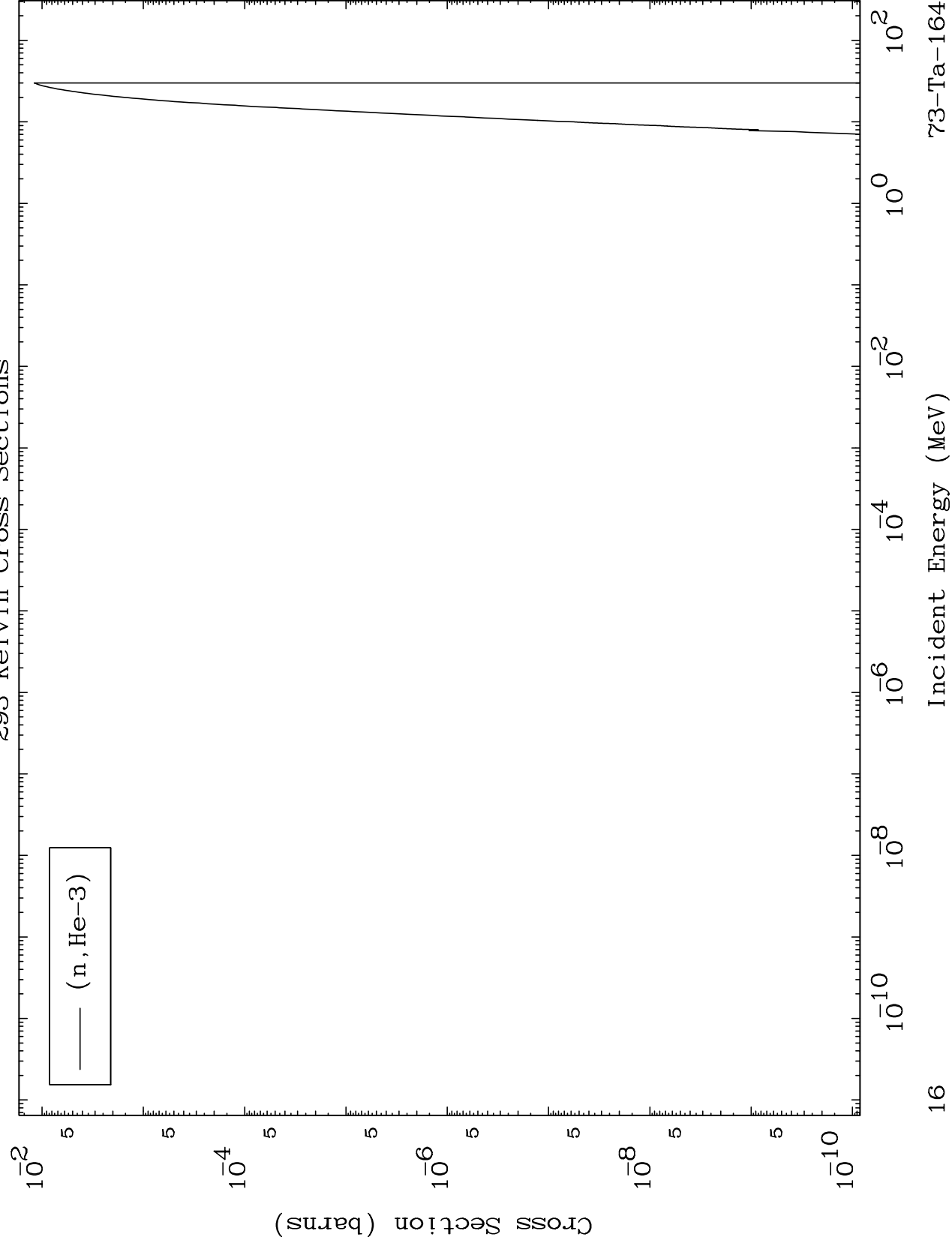
MAT 7277

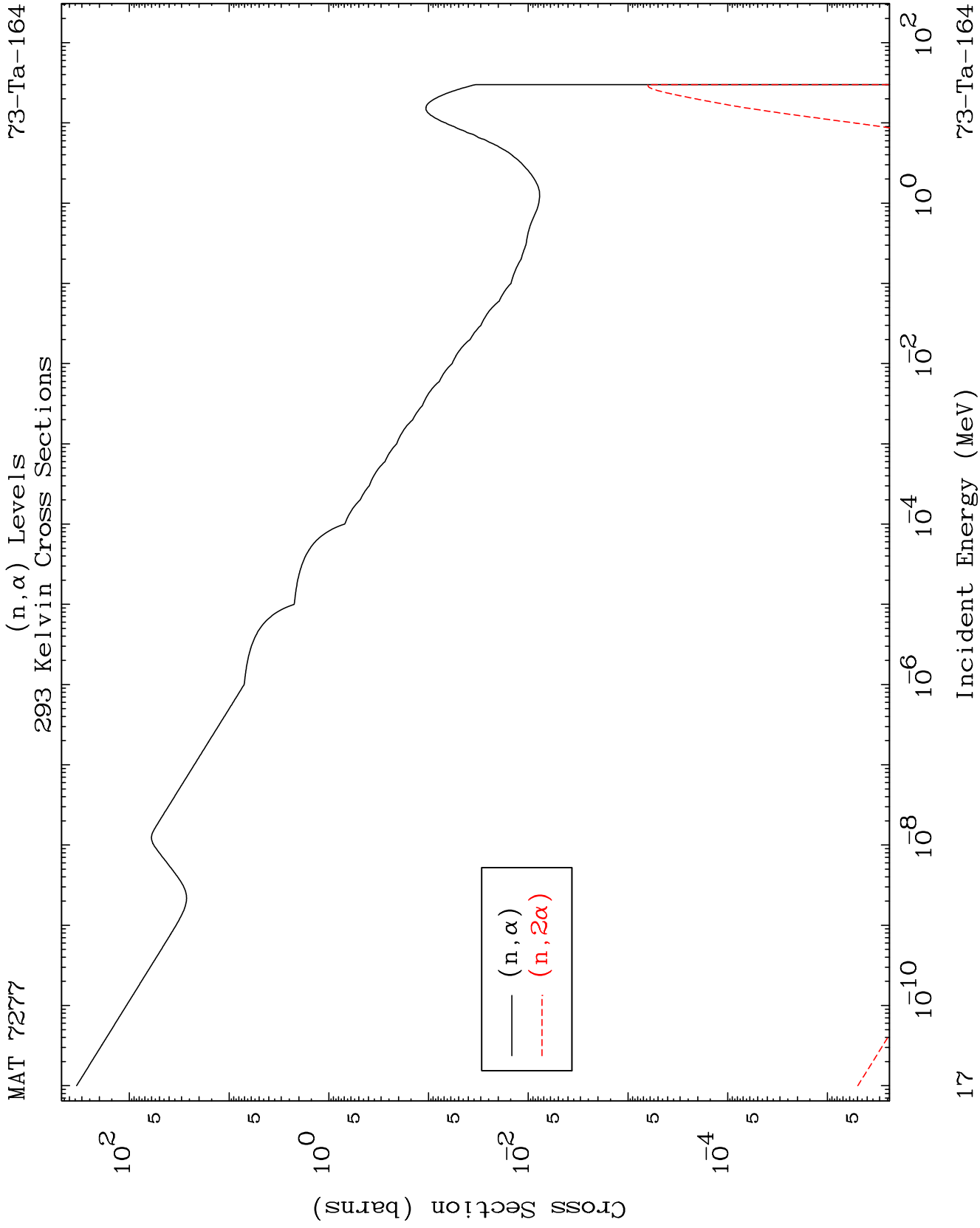
(n,t) Levels
293 Kelvin Cross Sections

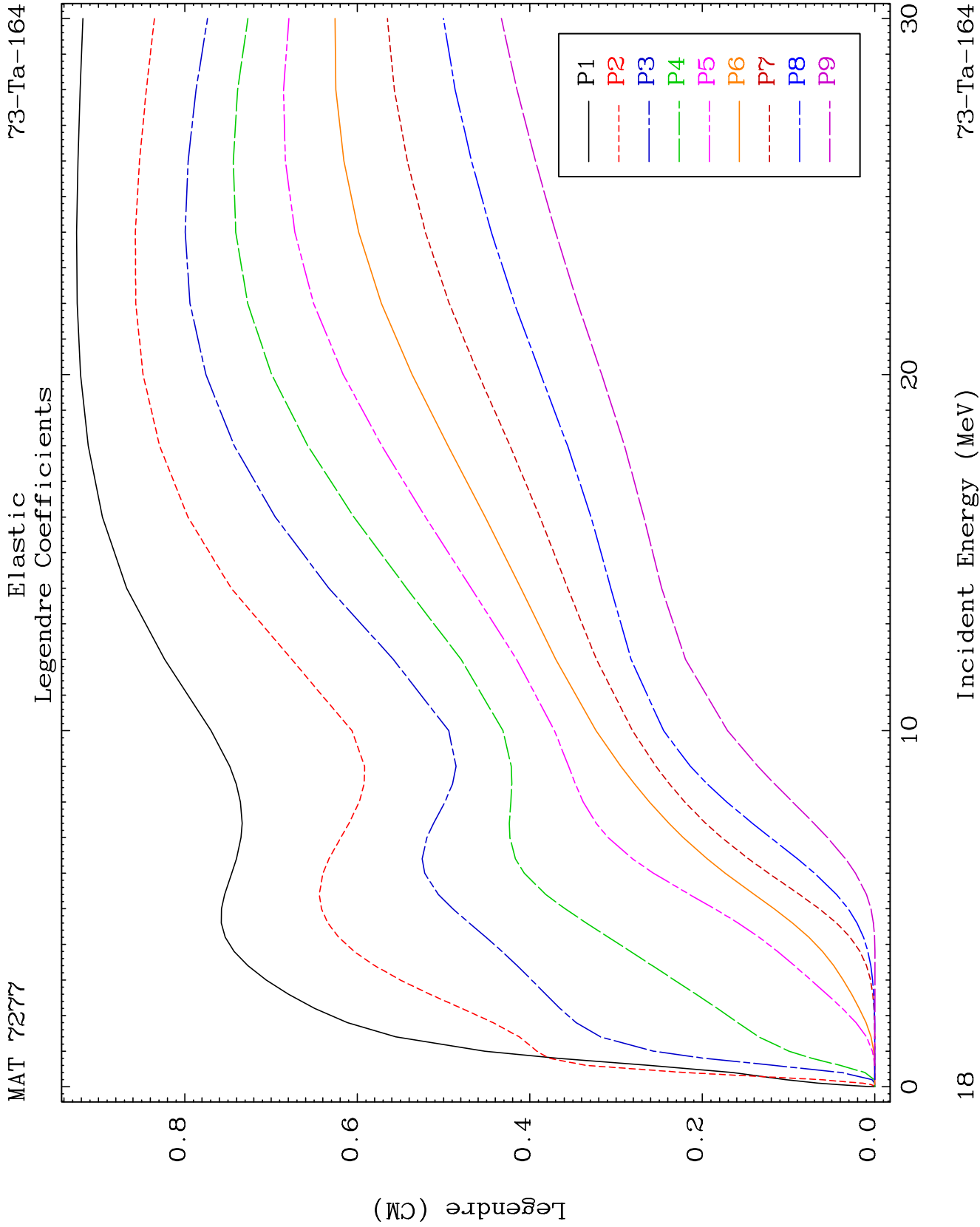
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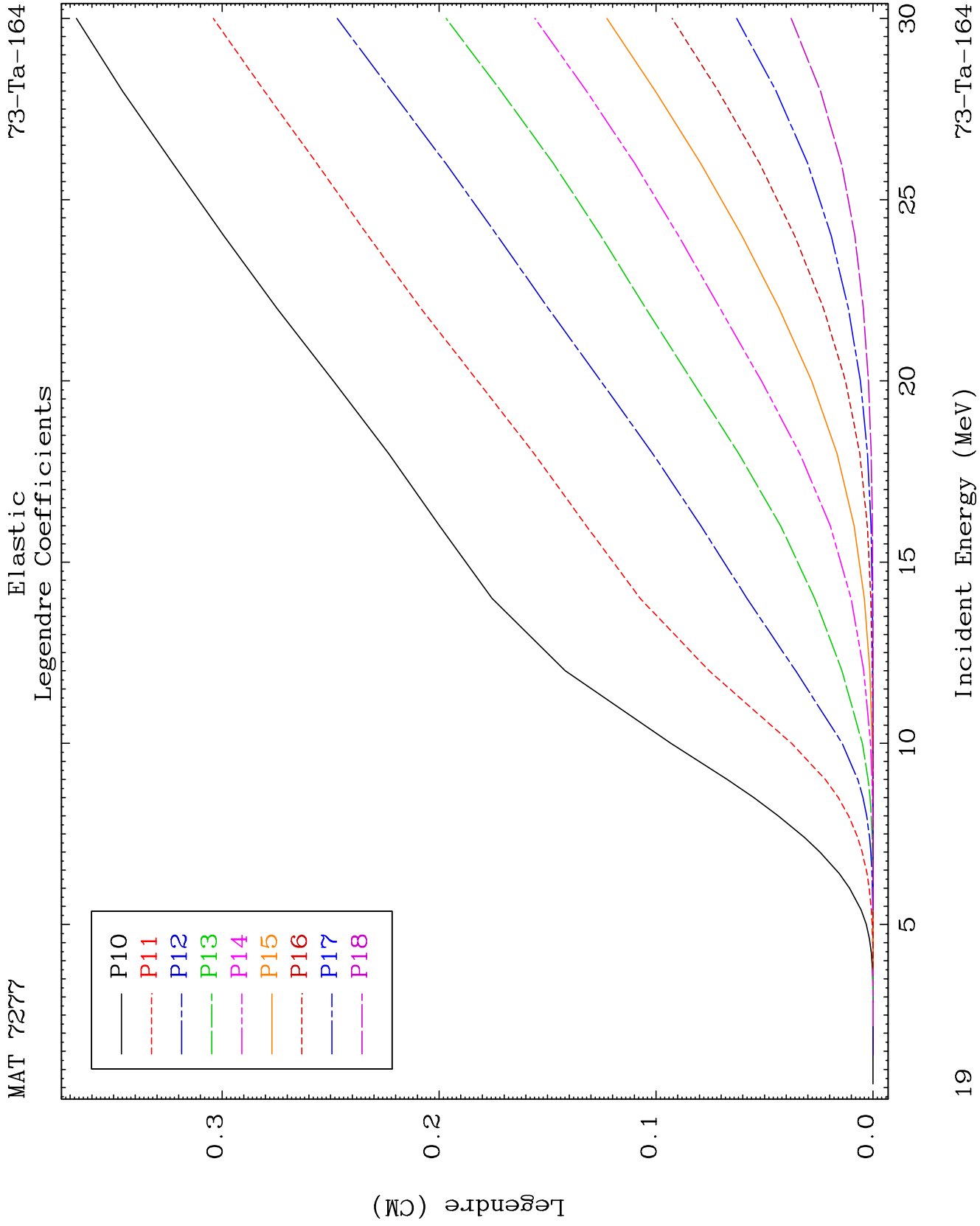


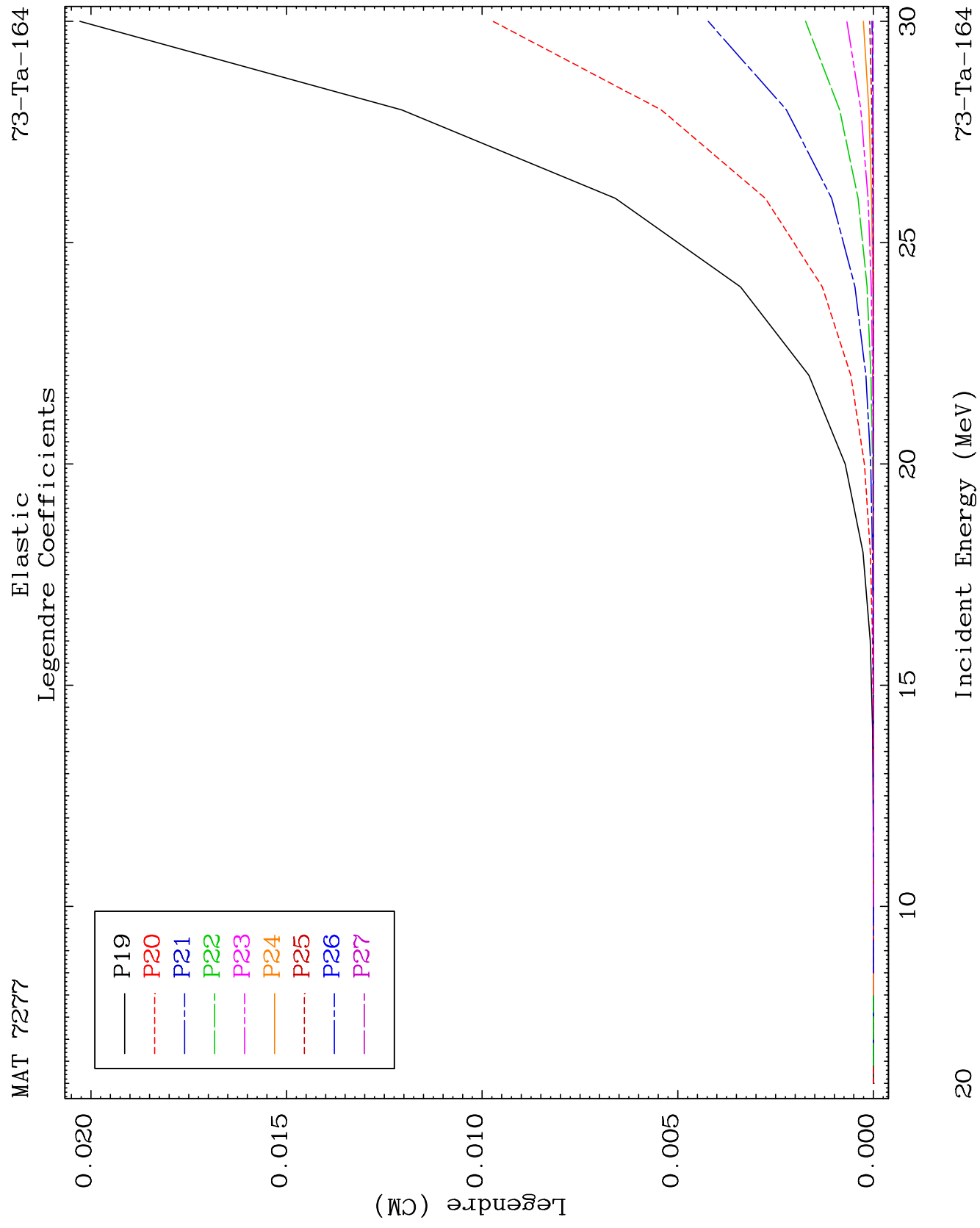
(n, He3) Levels
293 Kelvin Cross Sections

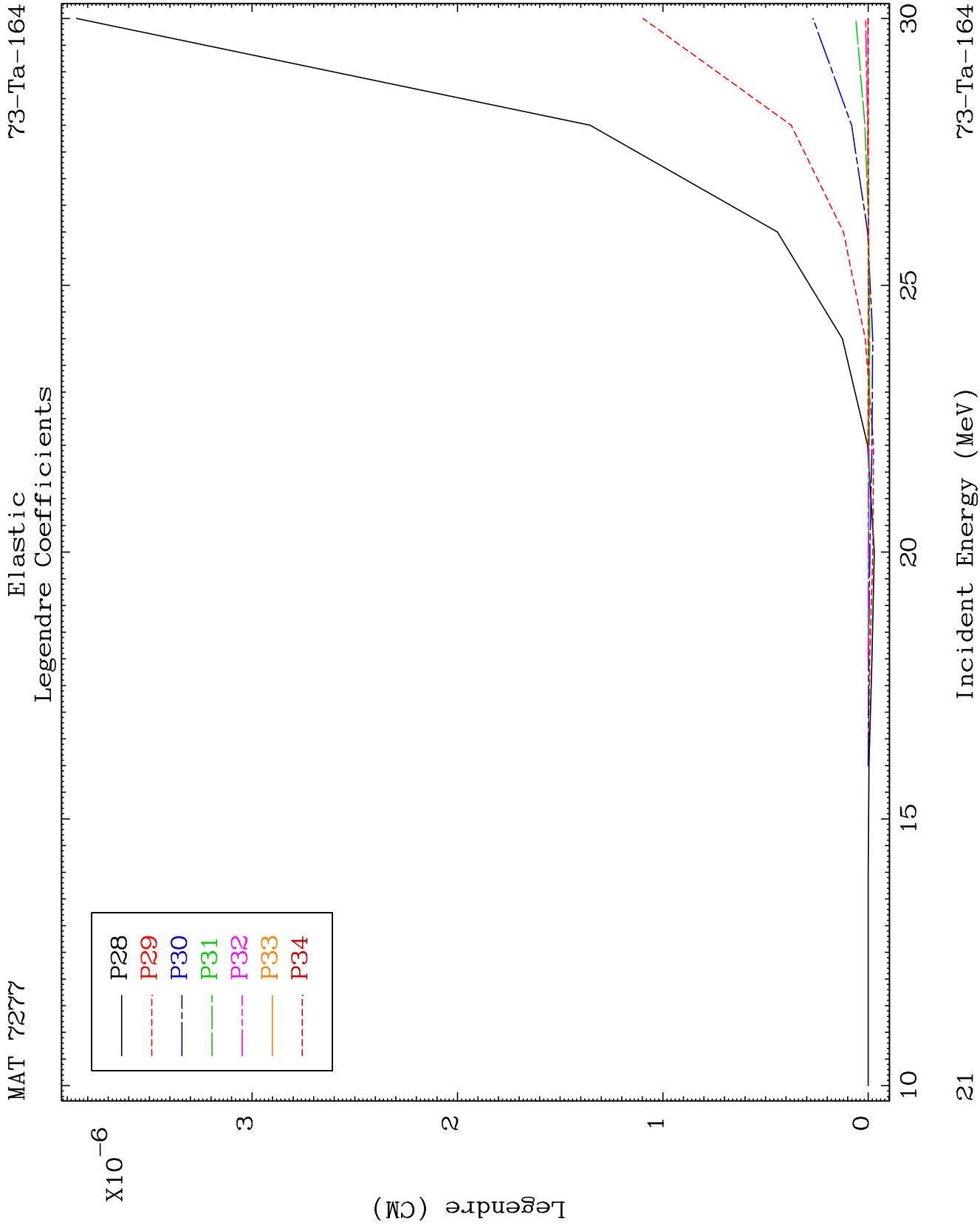


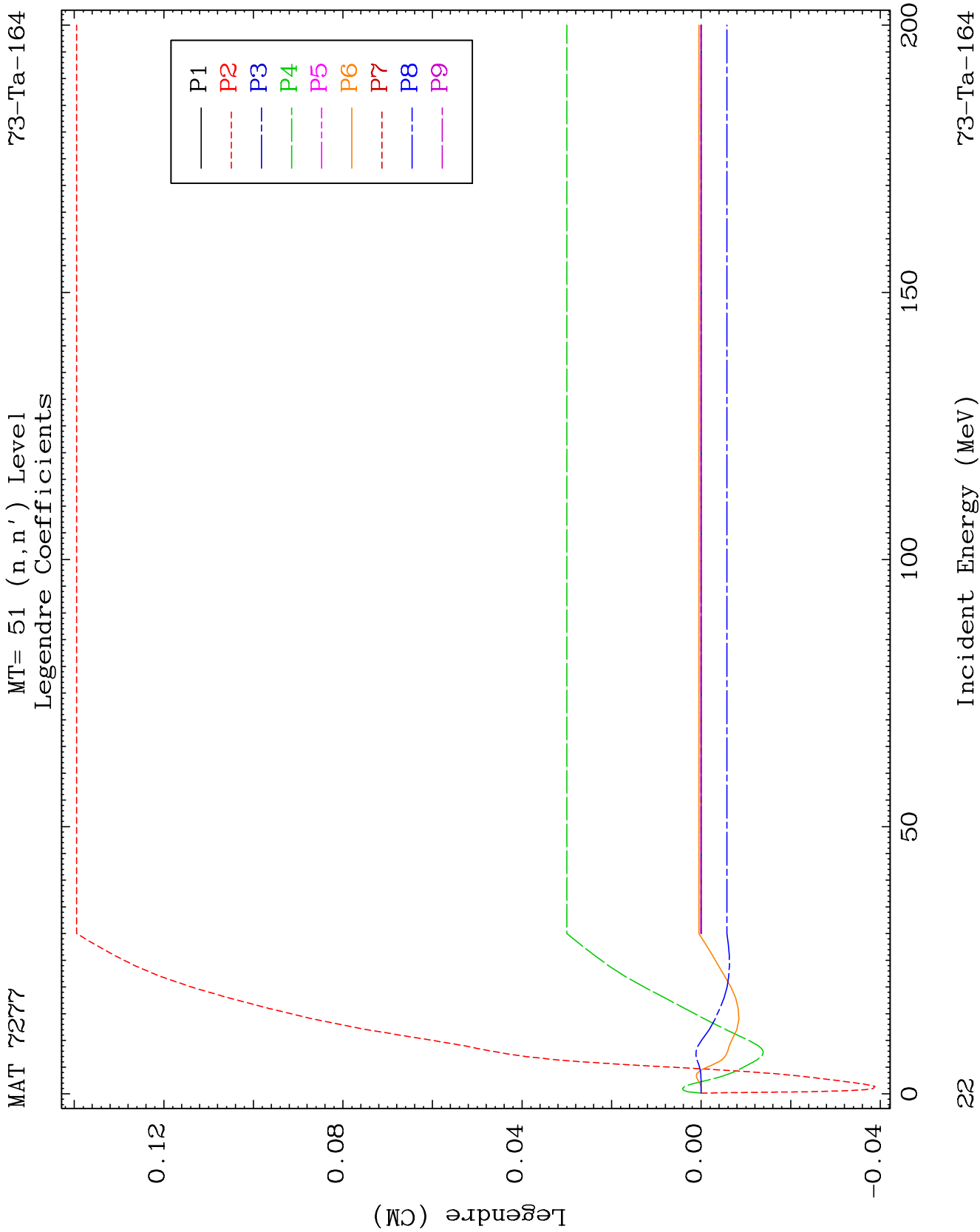


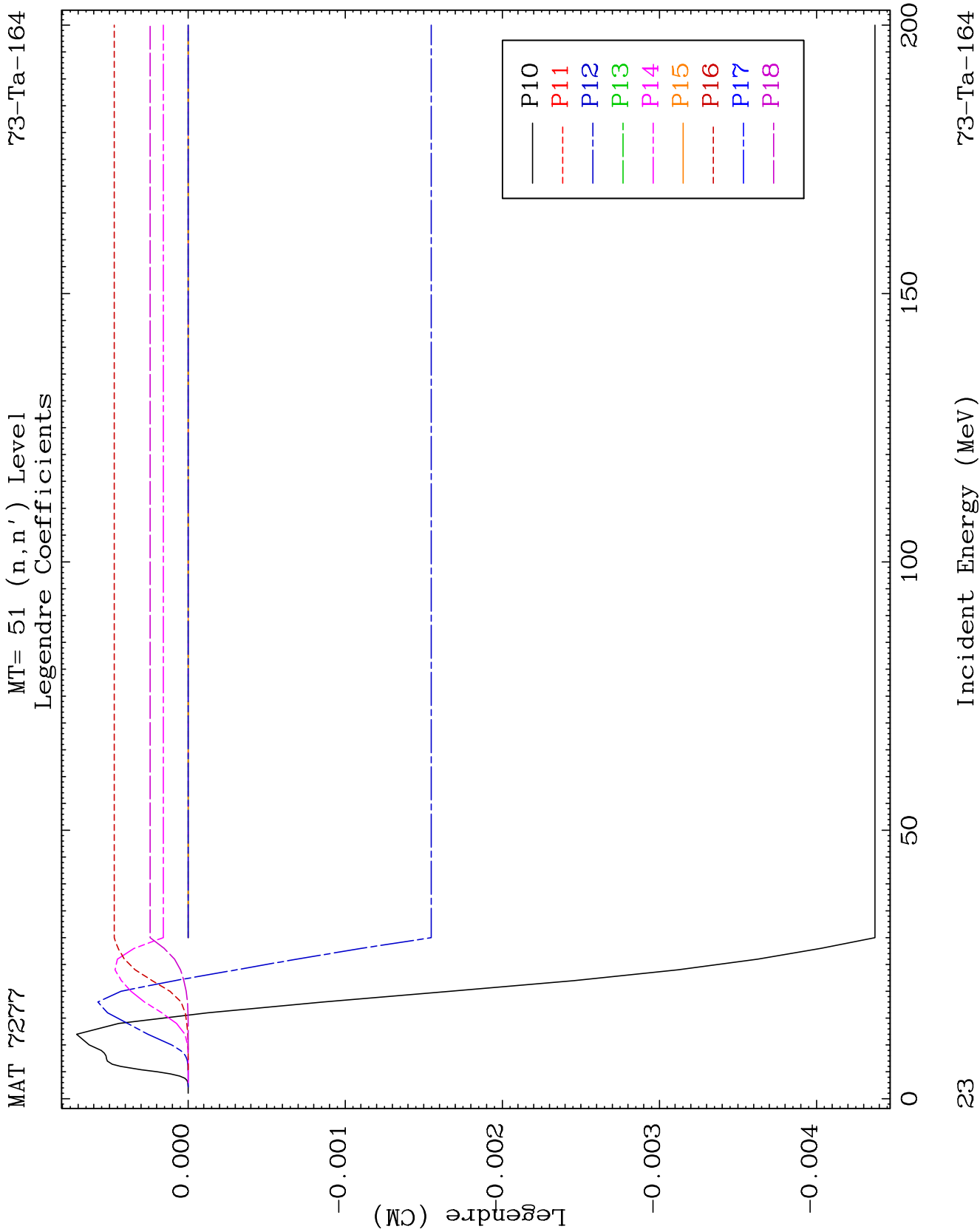


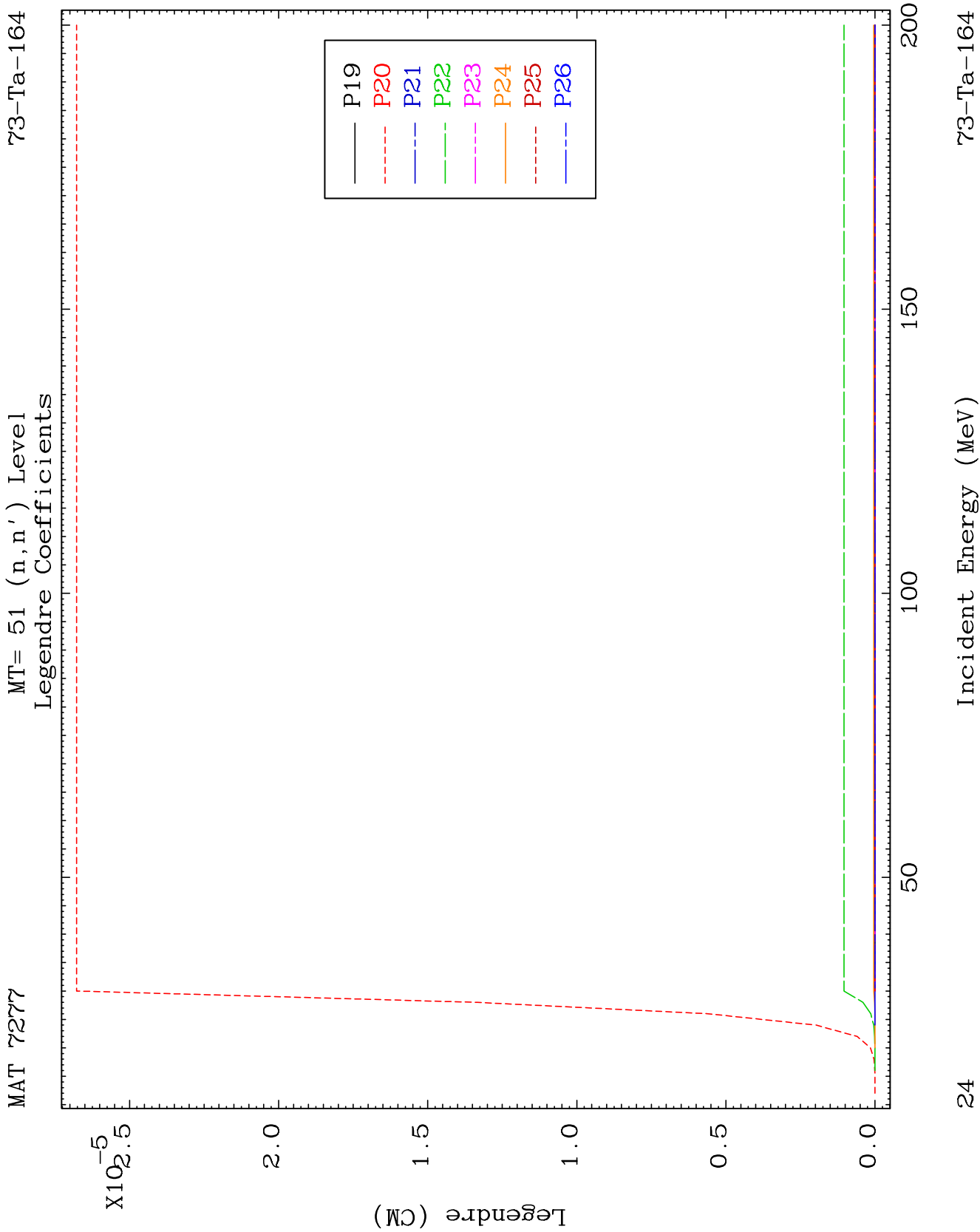








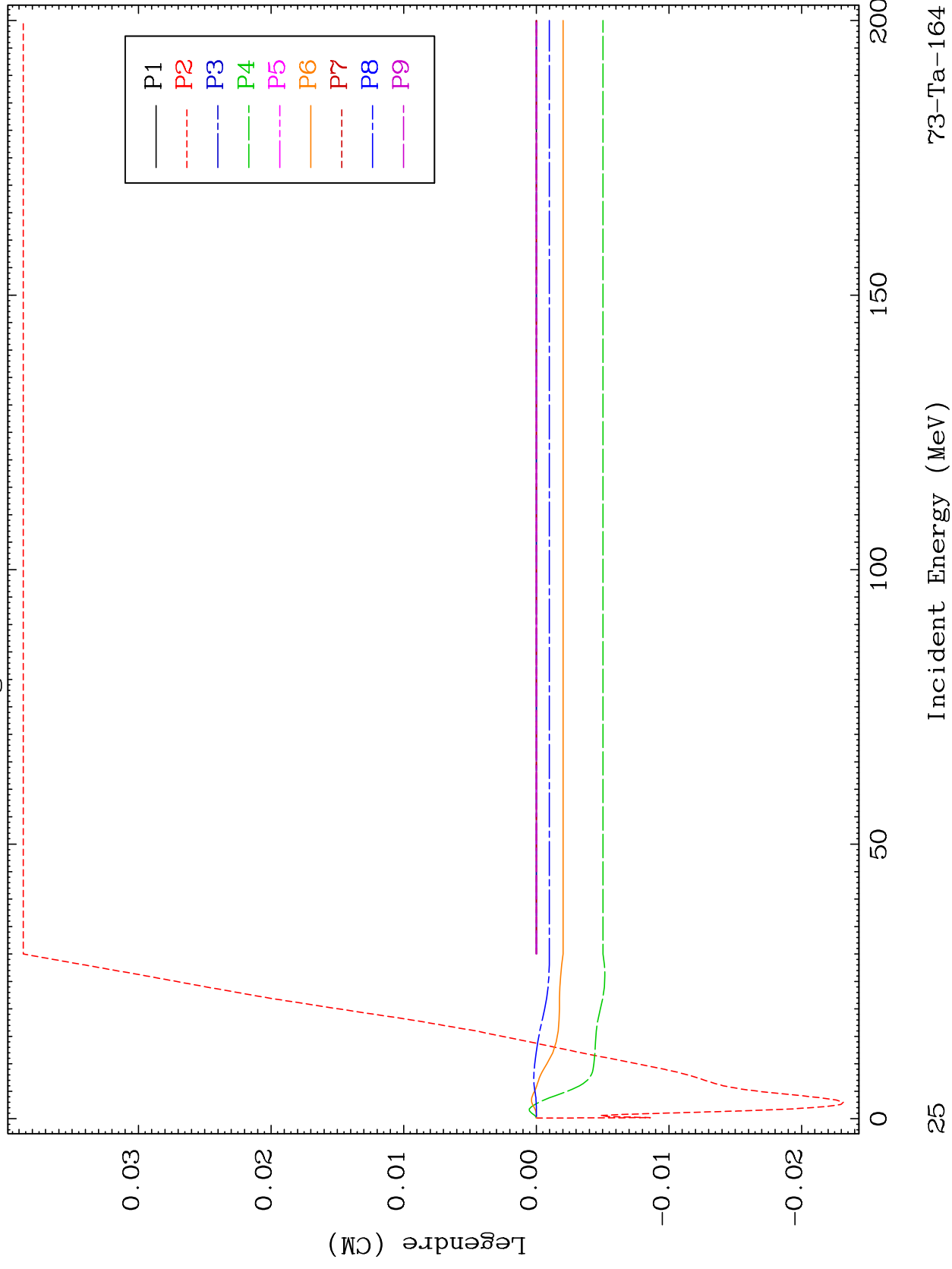


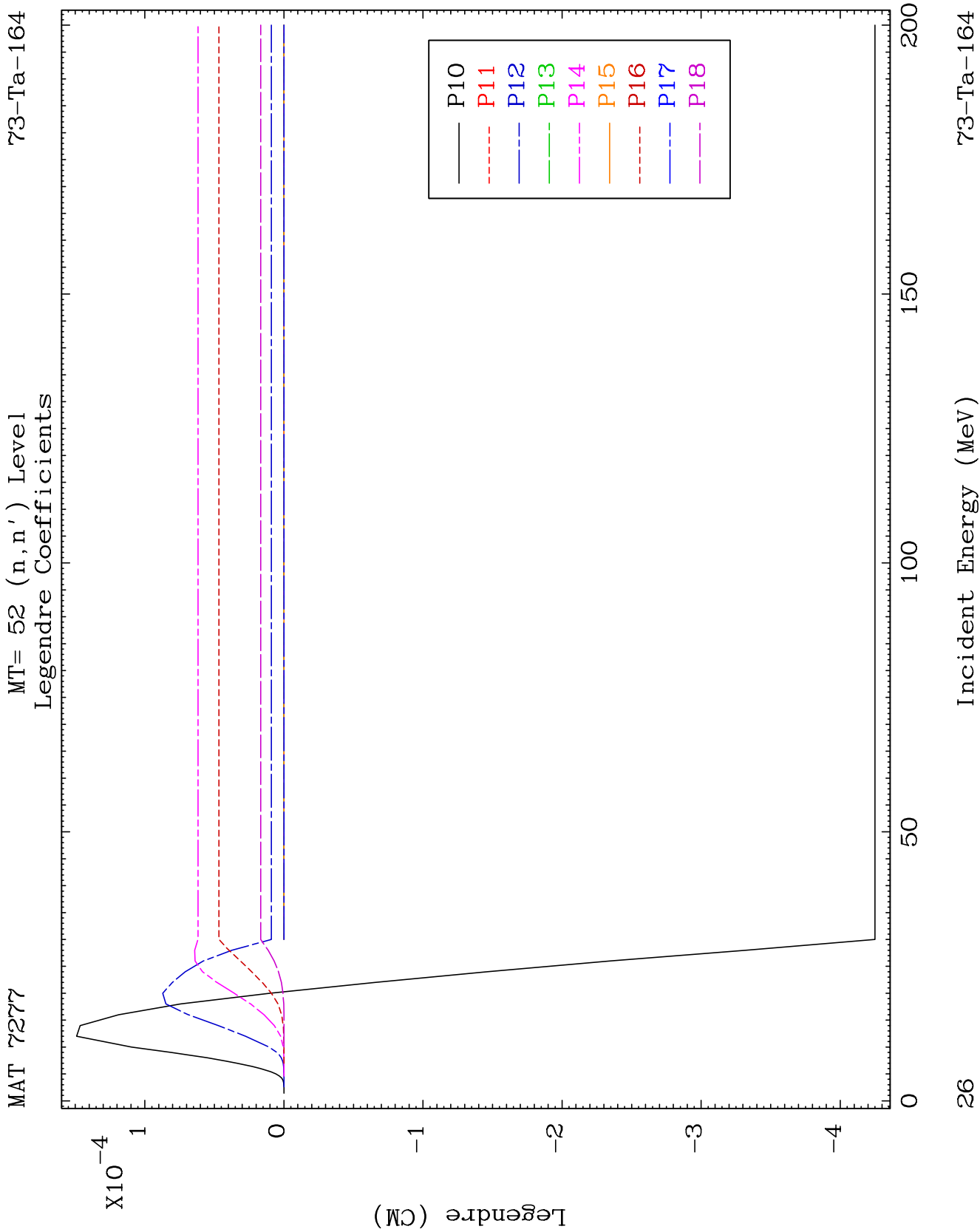


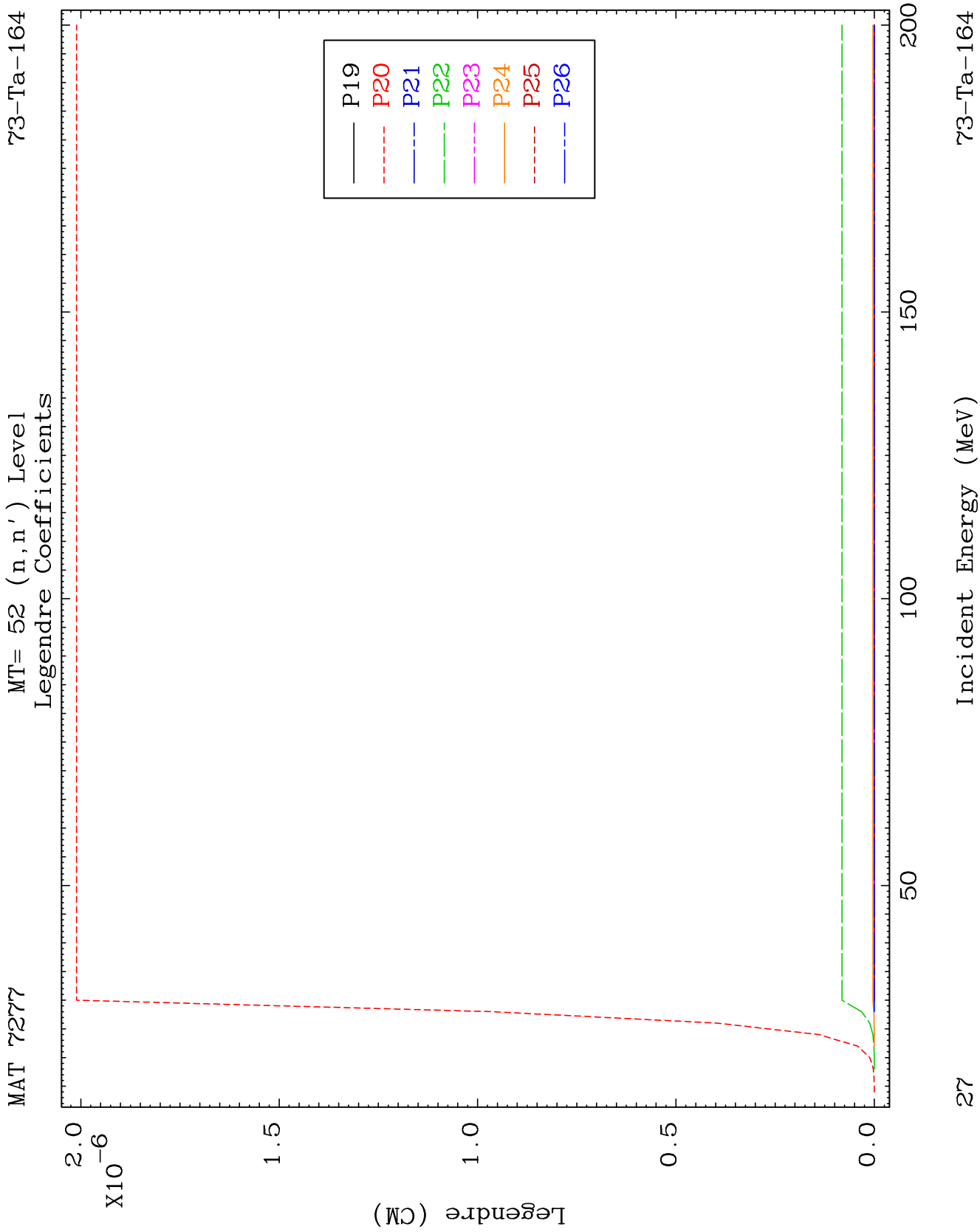
MAT 7277

MT= 52 (n, n') Level
Legendre Coefficients

73-Ta-164



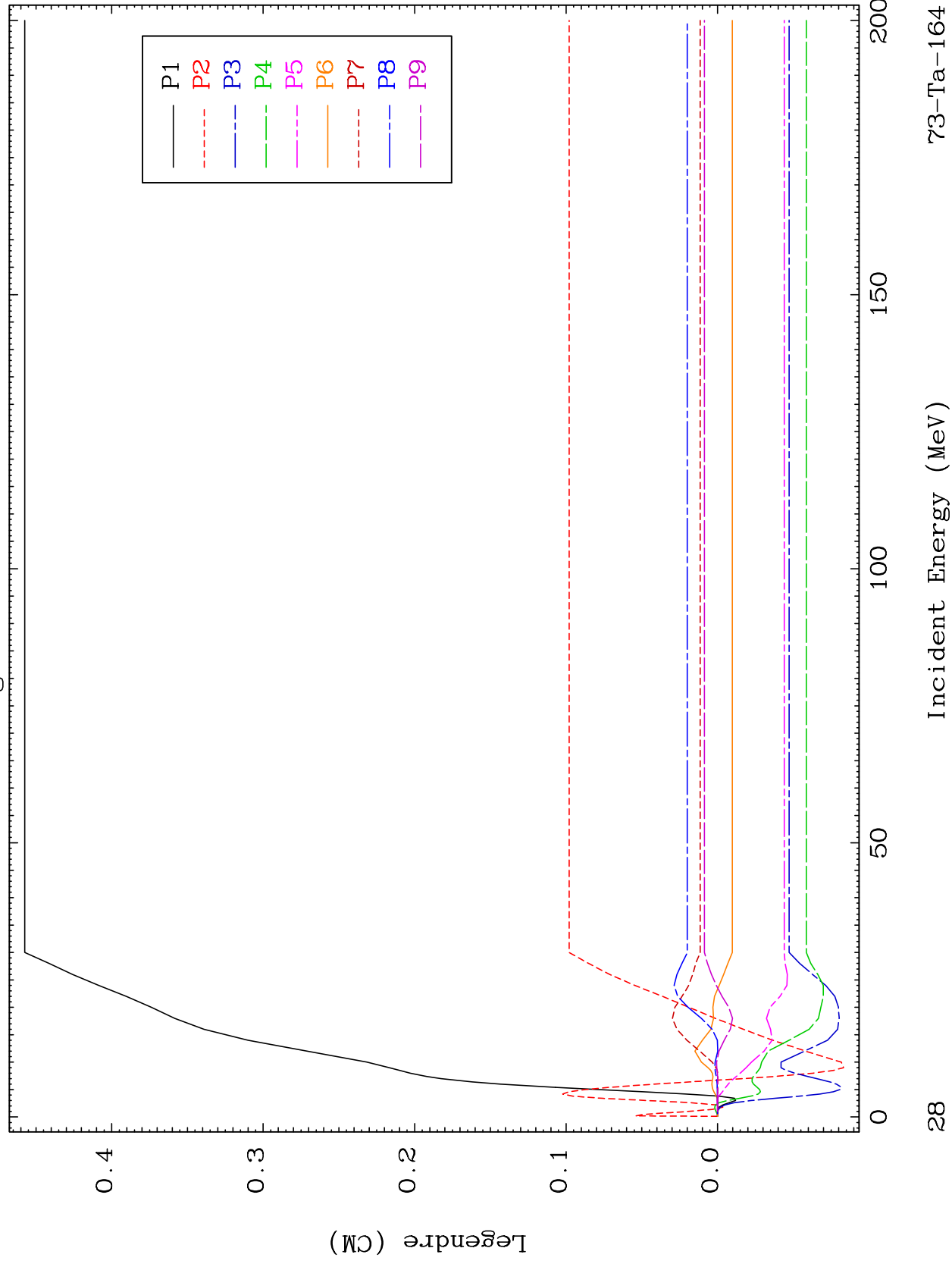


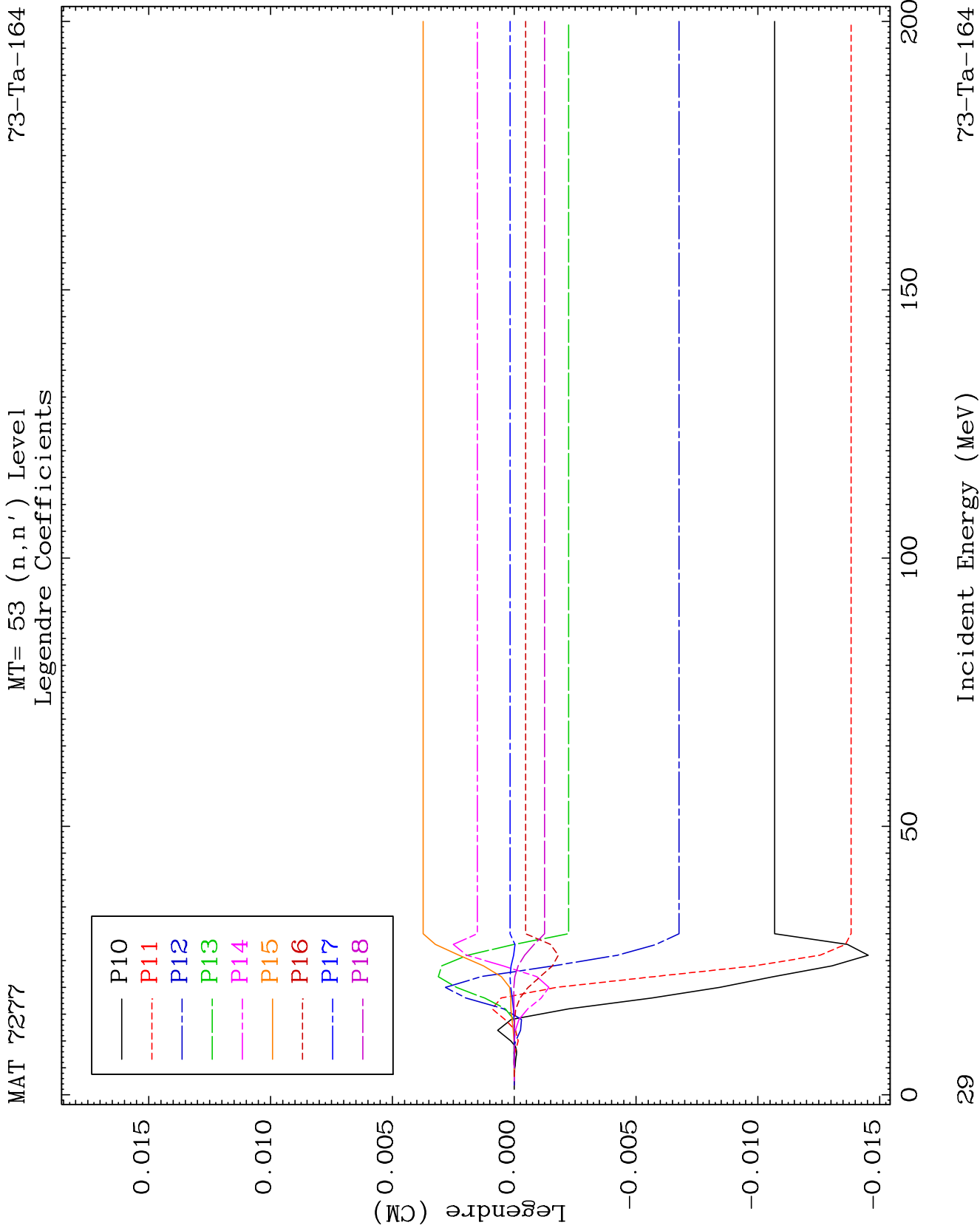


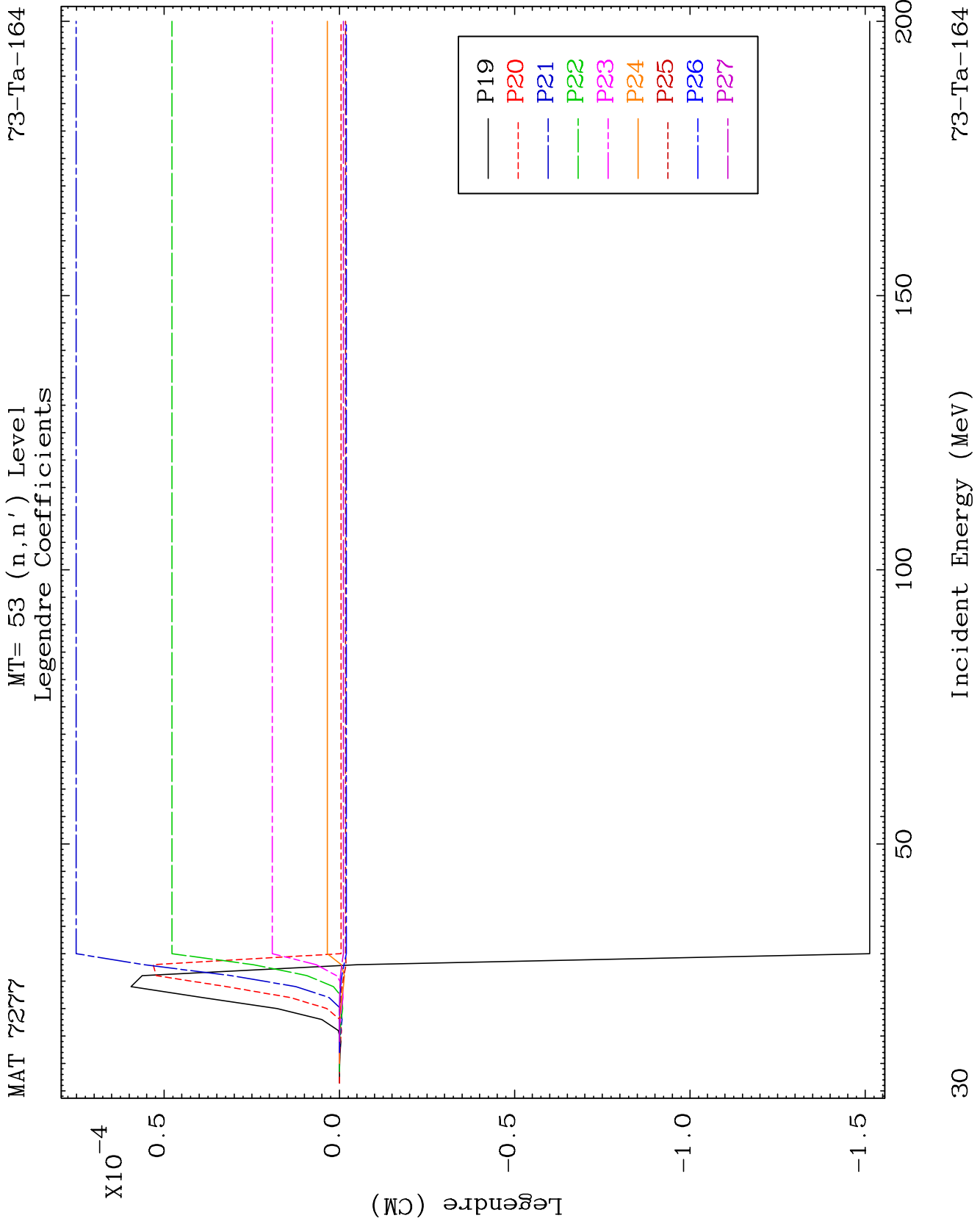
MAT 7277

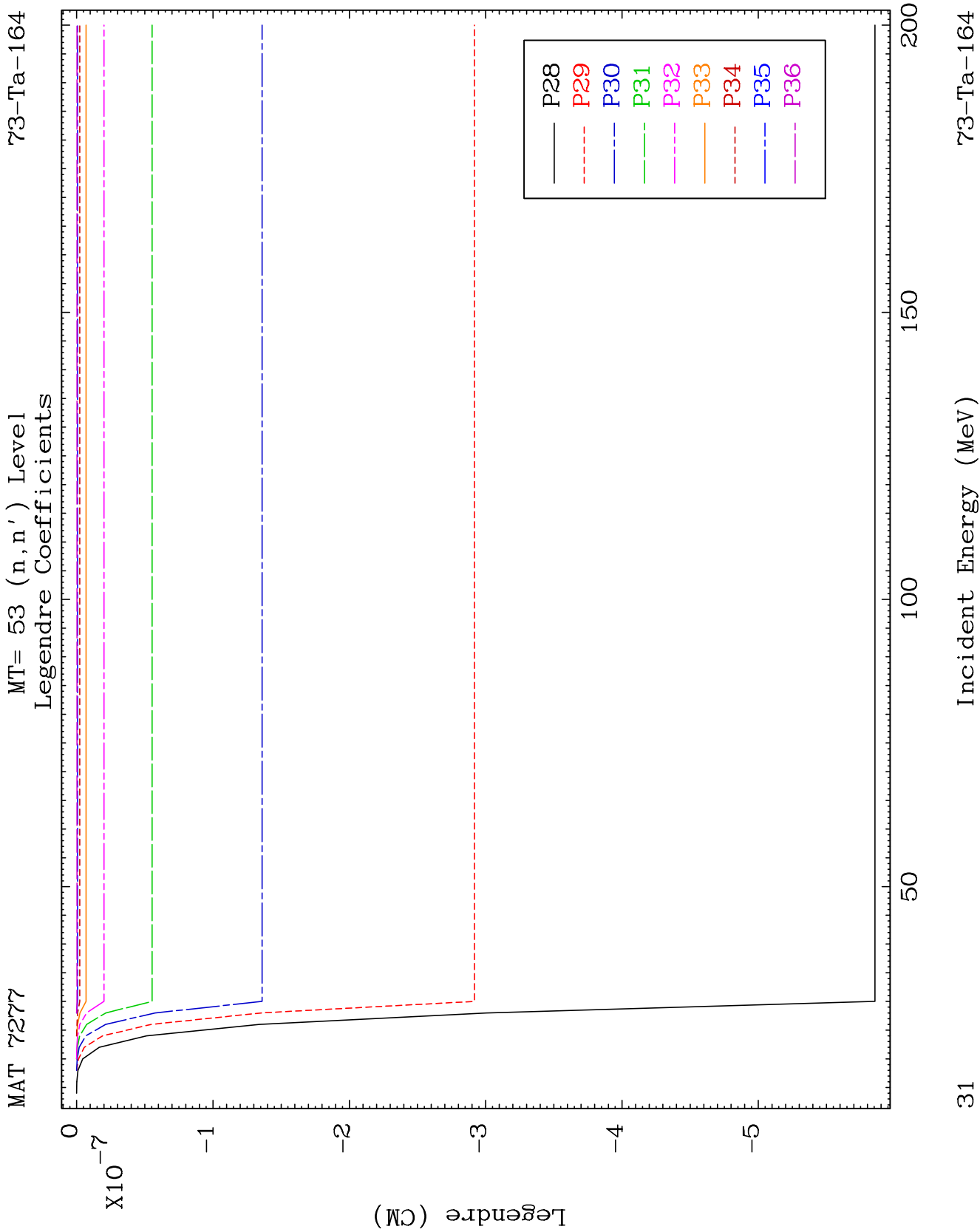
MT= 53 (n, n') Level
Legendre Coefficients

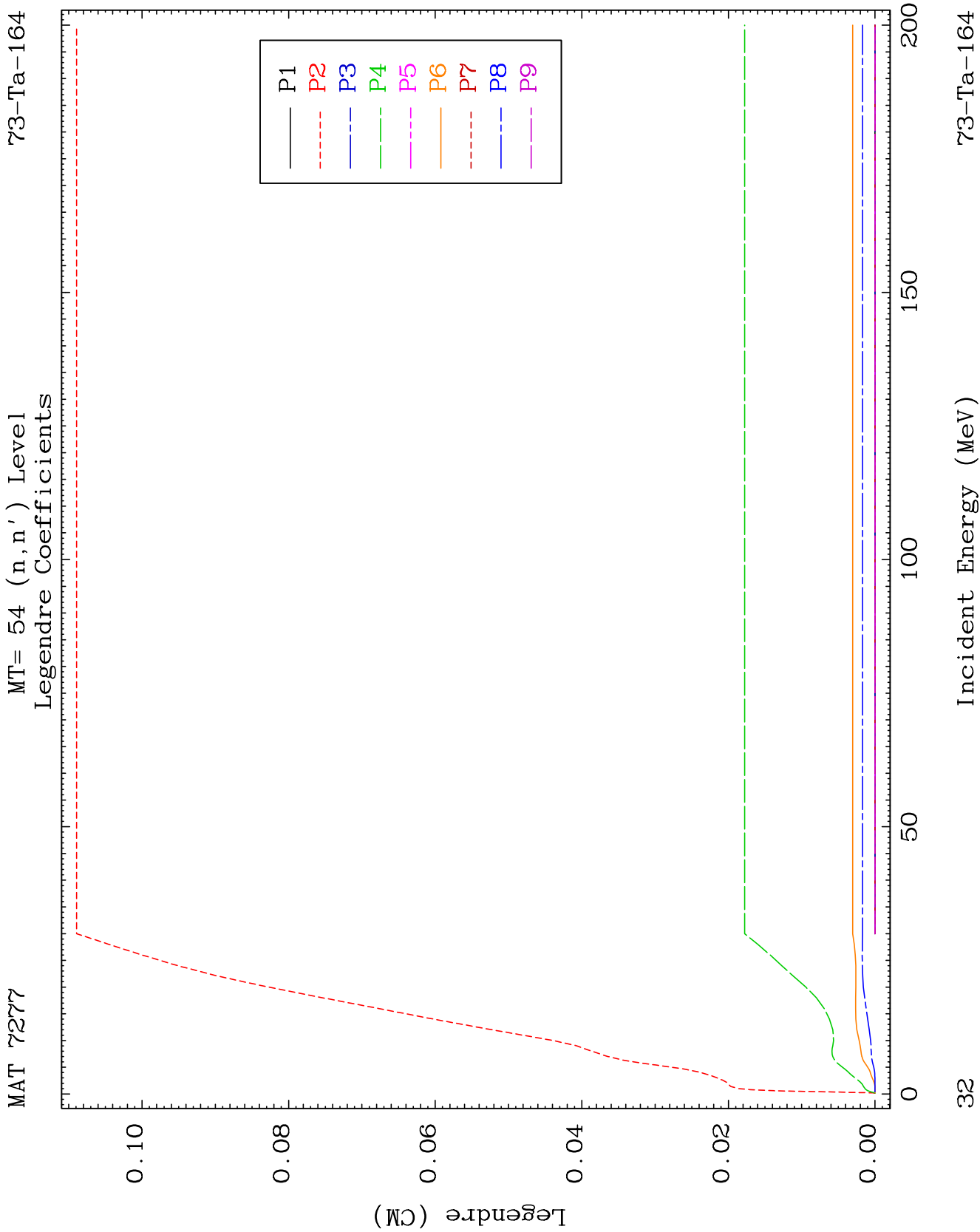
73-Ta-164

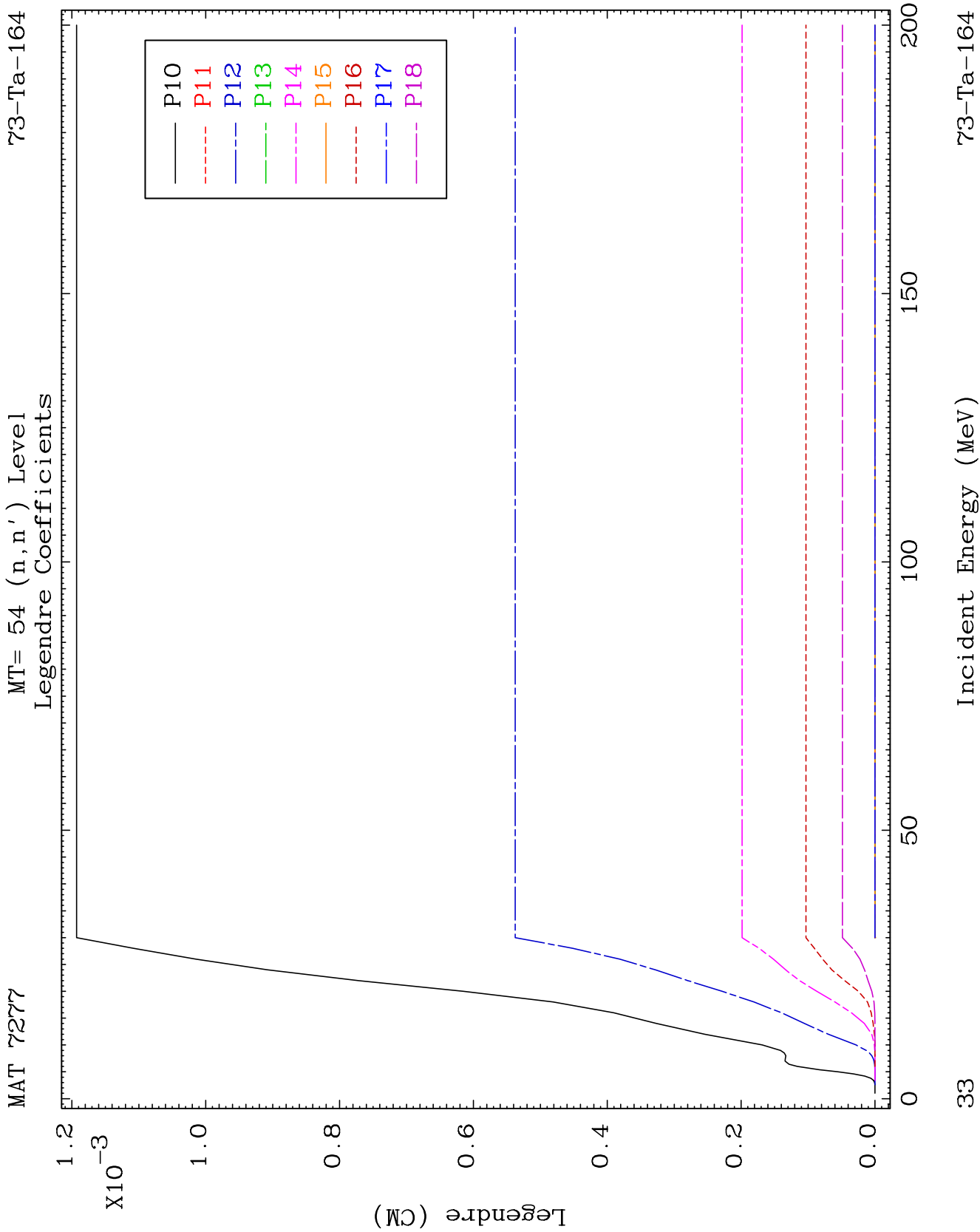








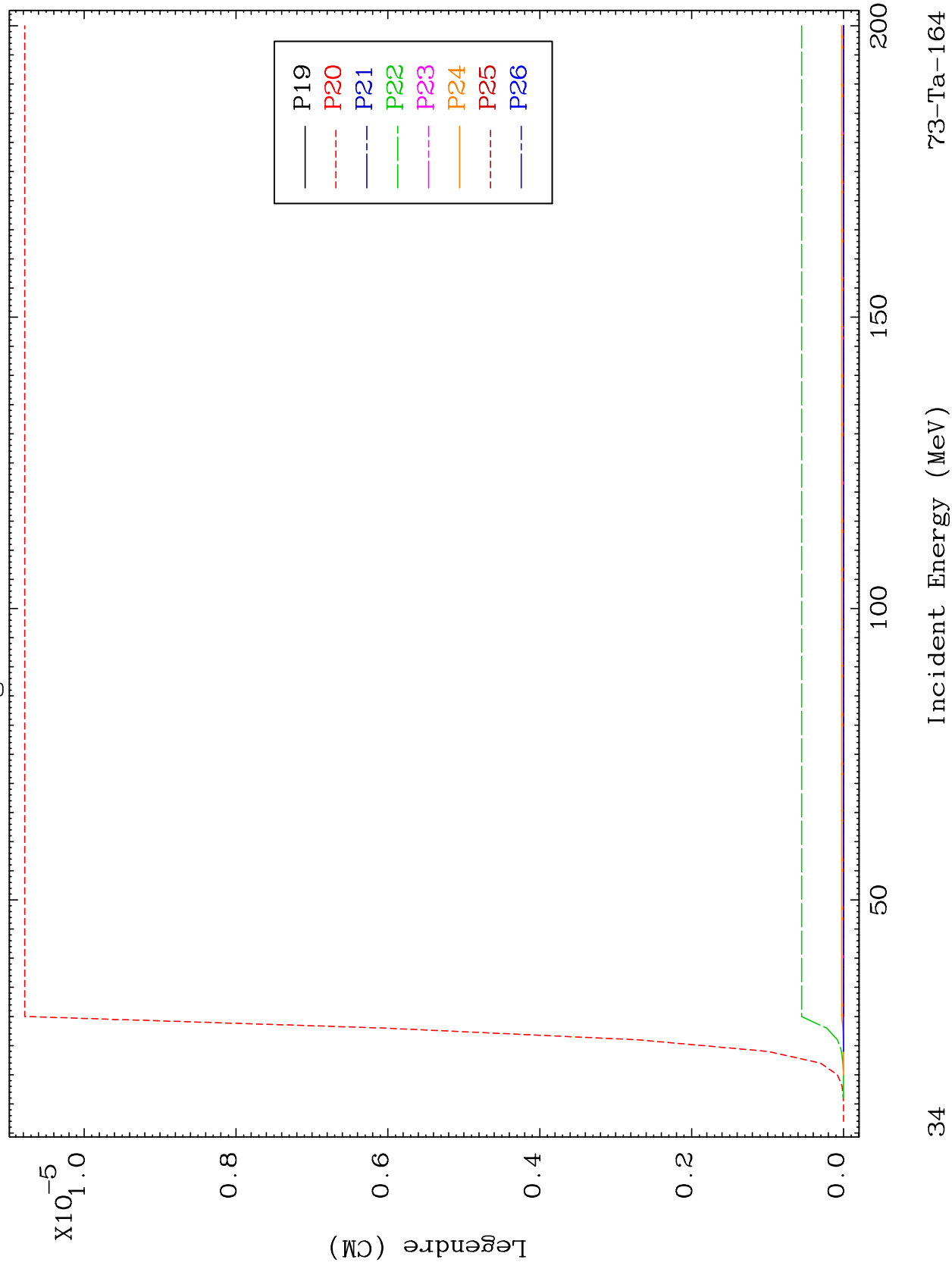




MAT 7277

MT= 54 (n, n') Level
Legendre Coefficients

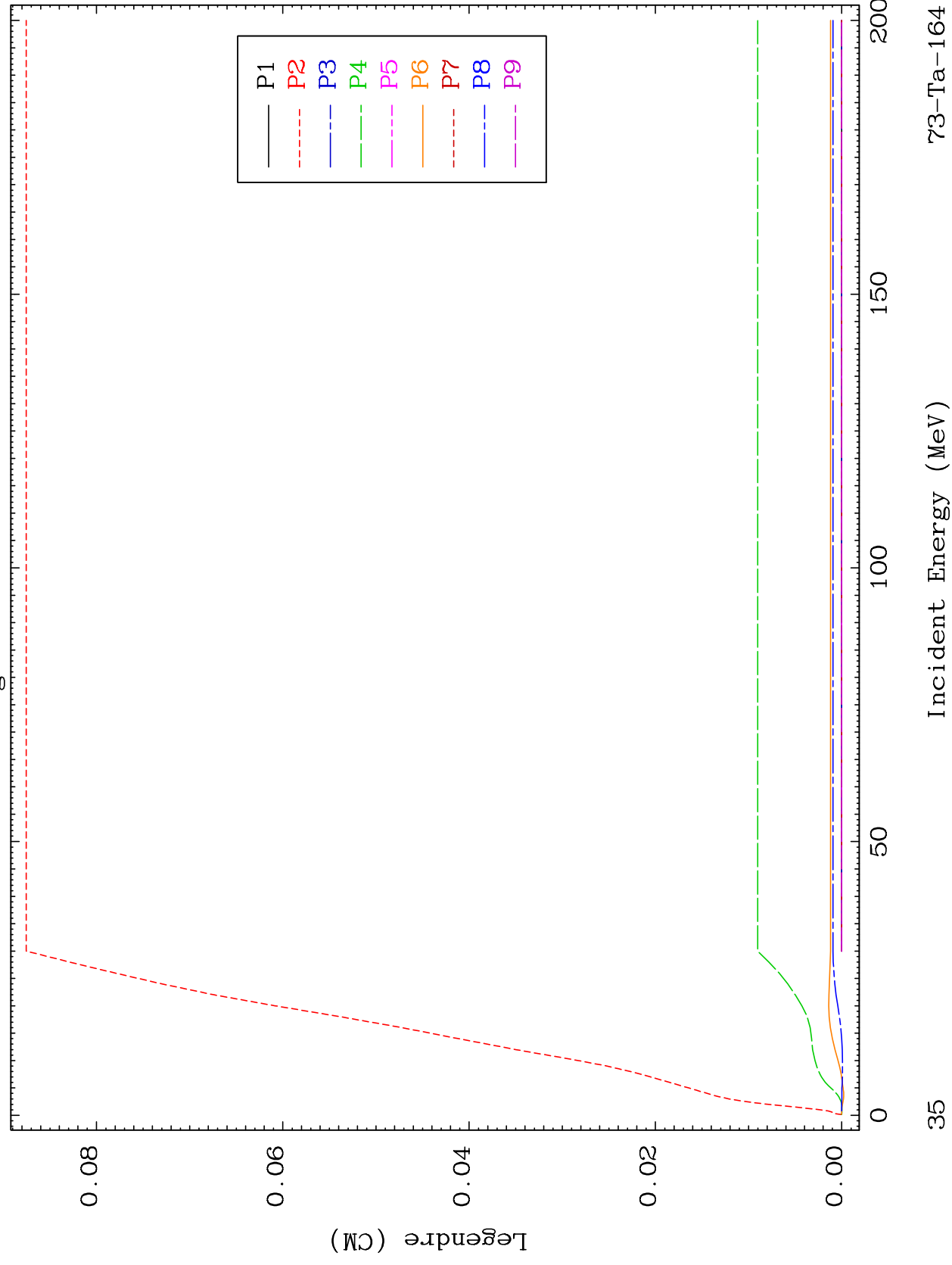
73-Ta-164



MAT 7277

MT= 55 (n, n') Level
Legendre Coefficients

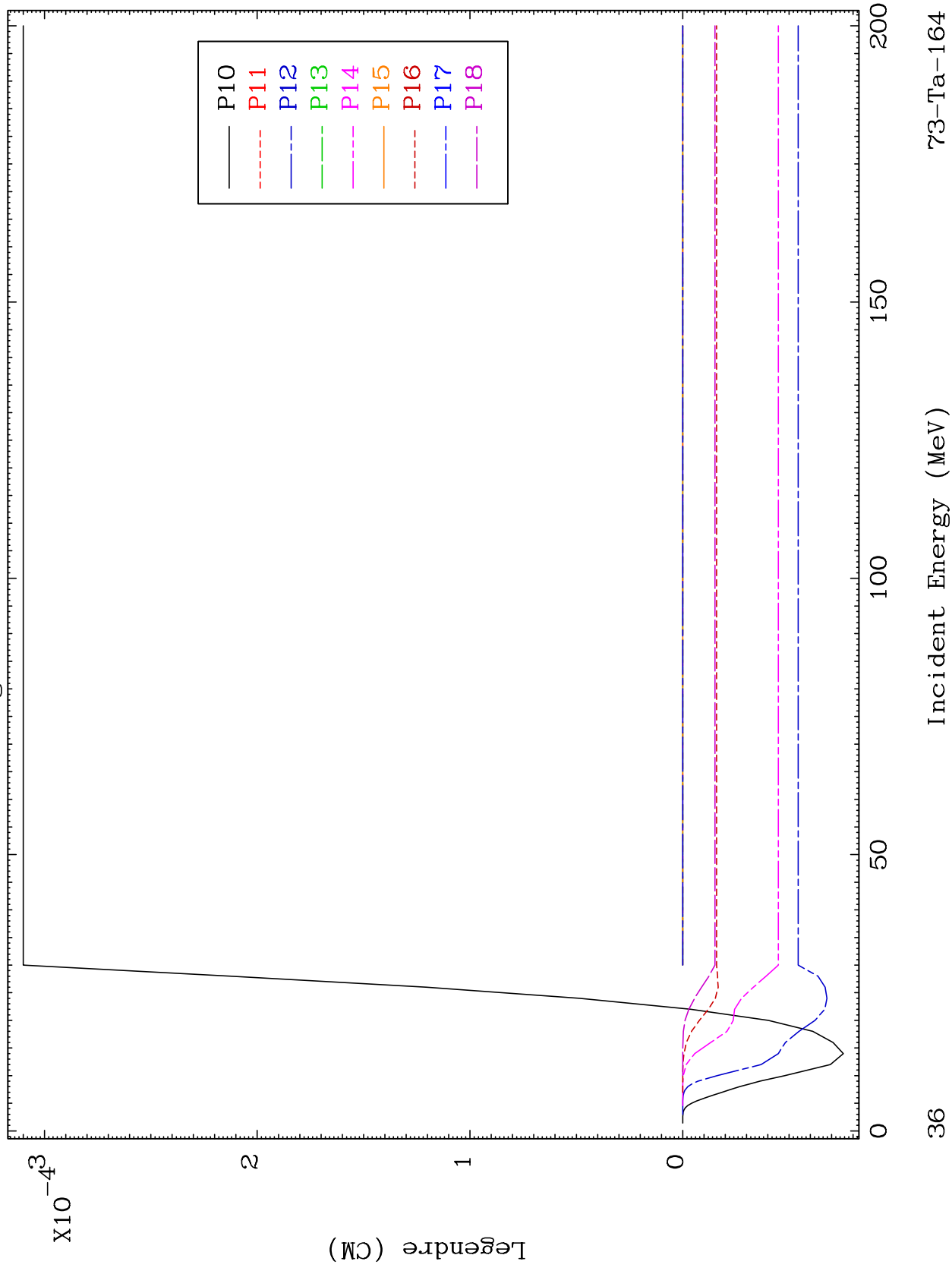
73-Ta-164

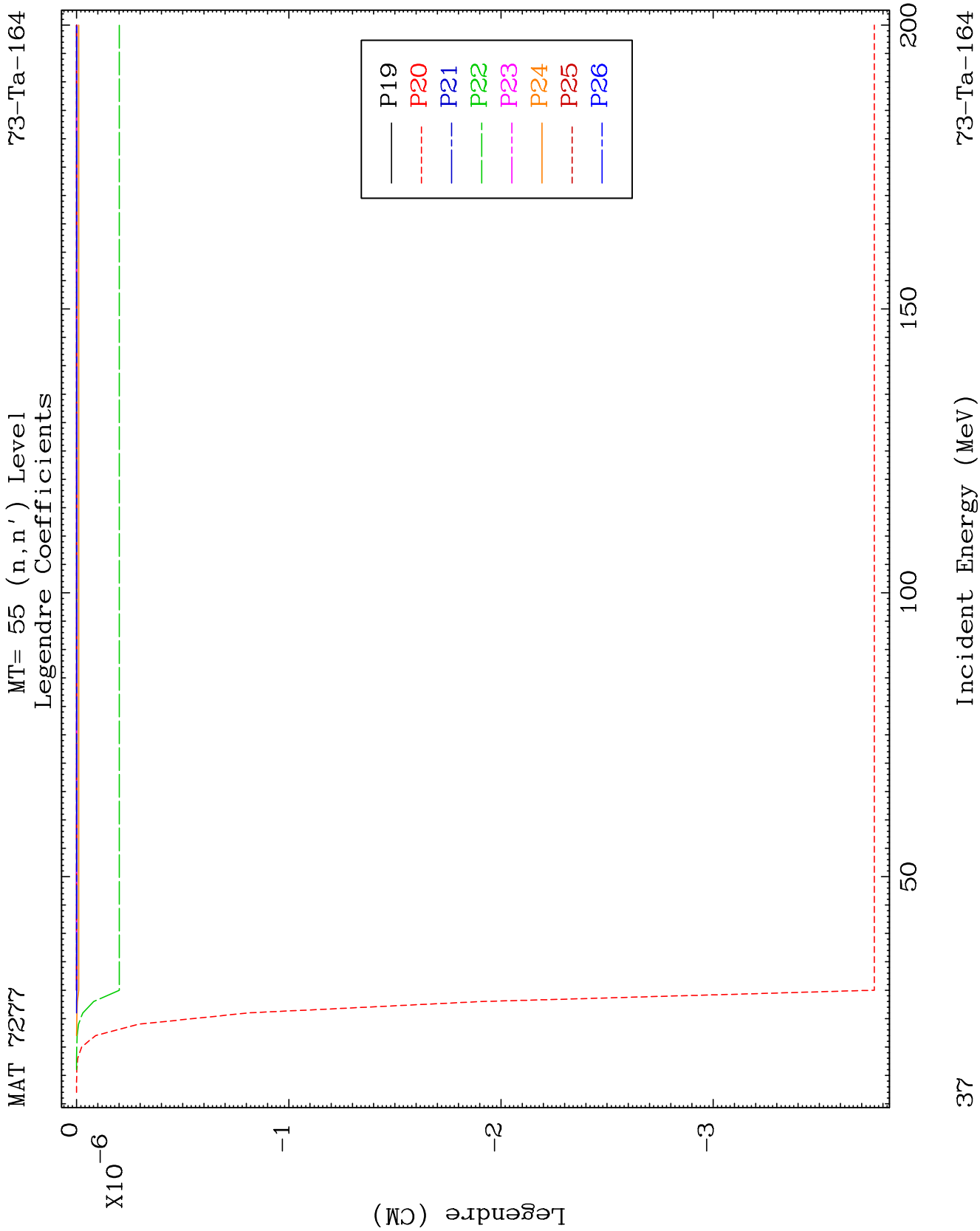


MAT 7277

MT= 55 (n, n') Level
Legendre Coefficients

73-Ta-164

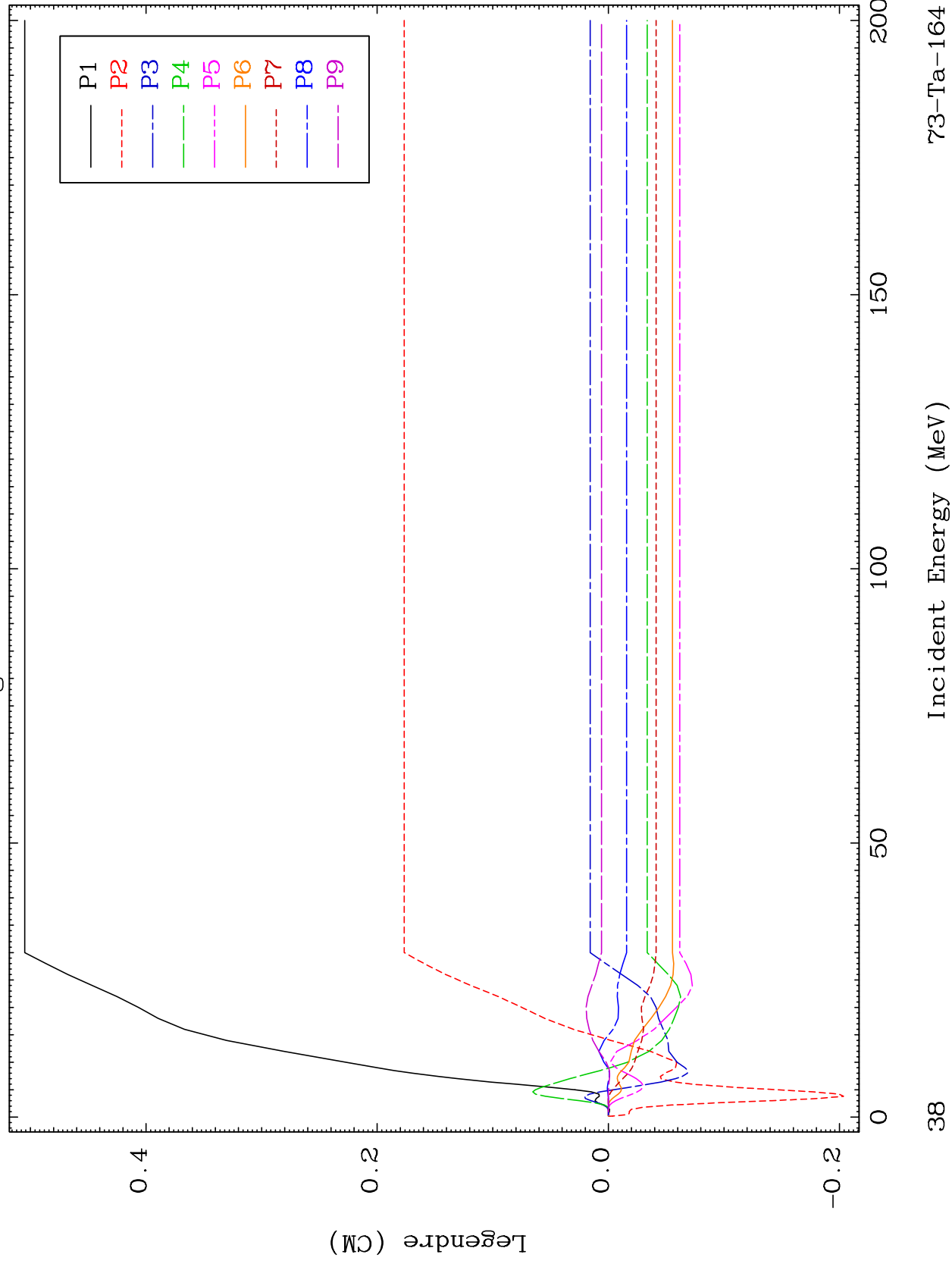


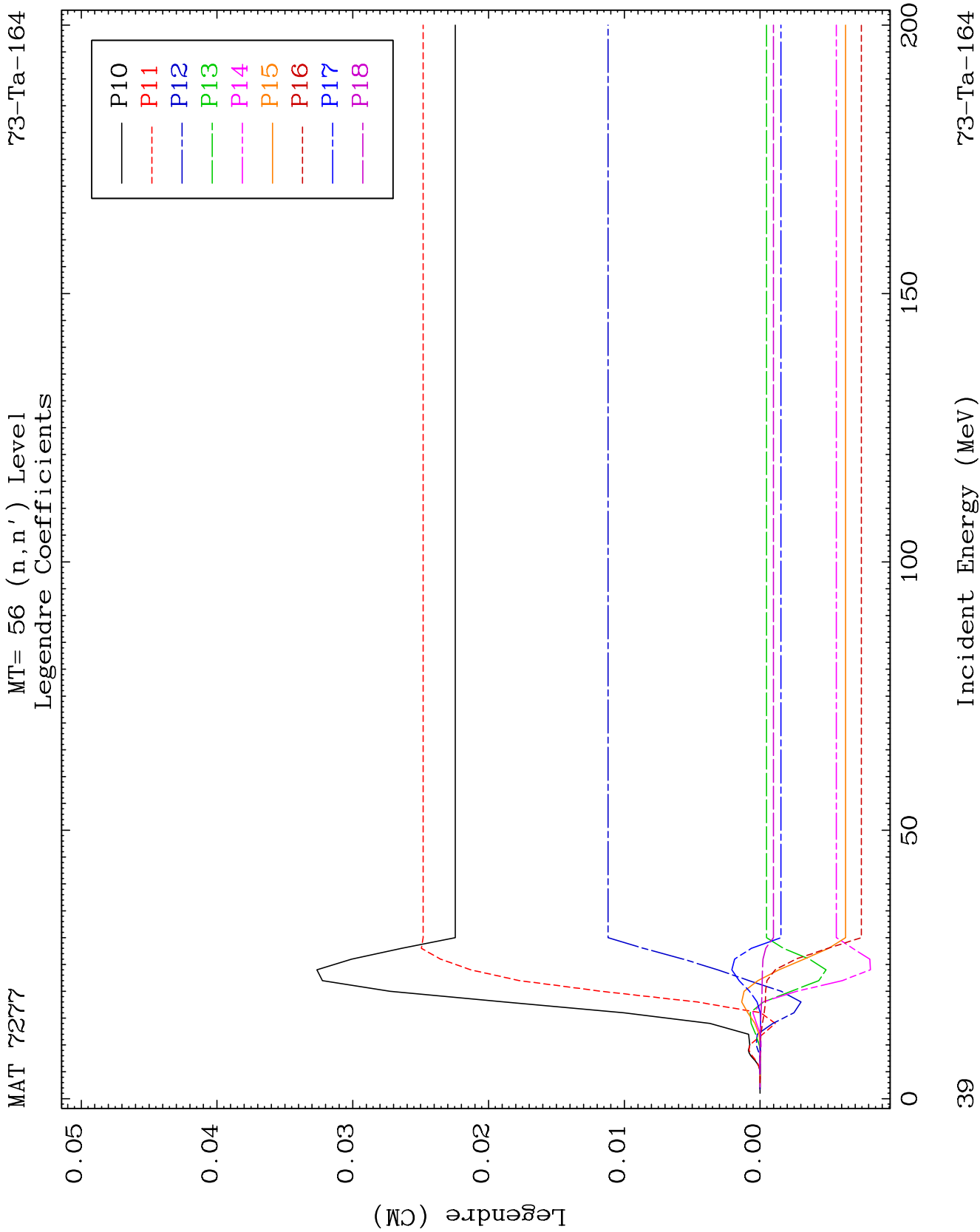


MAT 7277

MT= 56 (n, n') Level
Legendre Coefficients

73-Ta-164

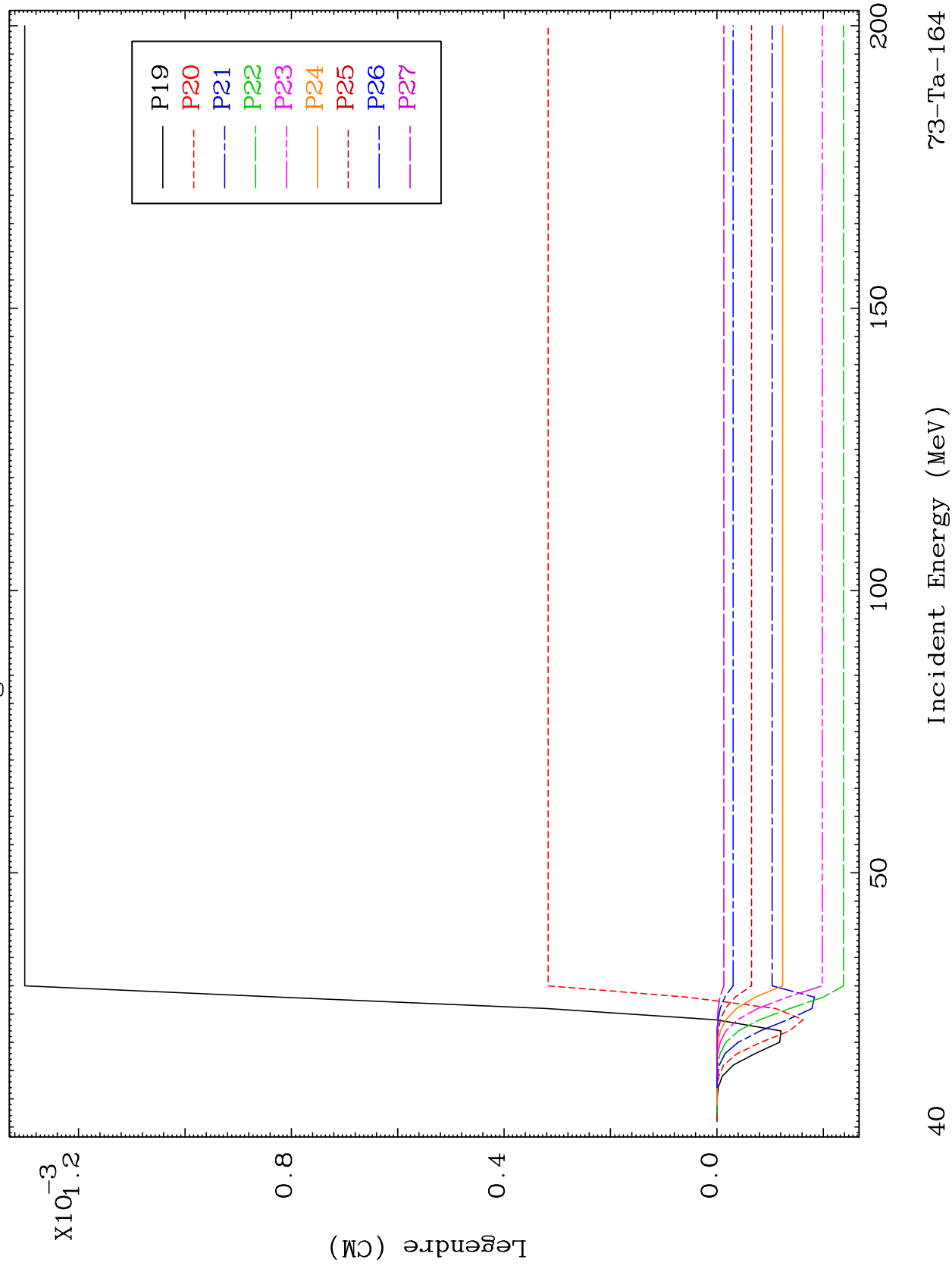


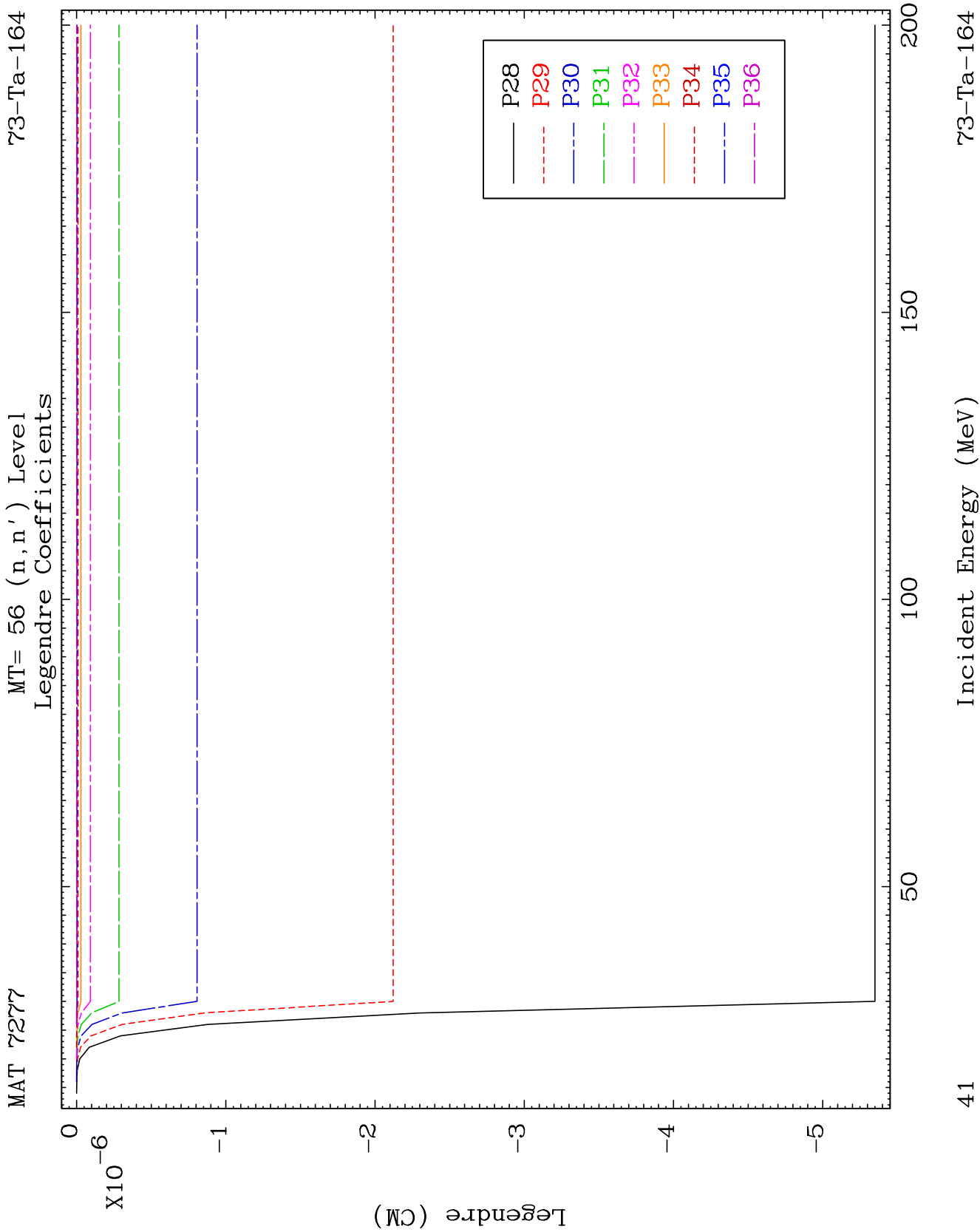


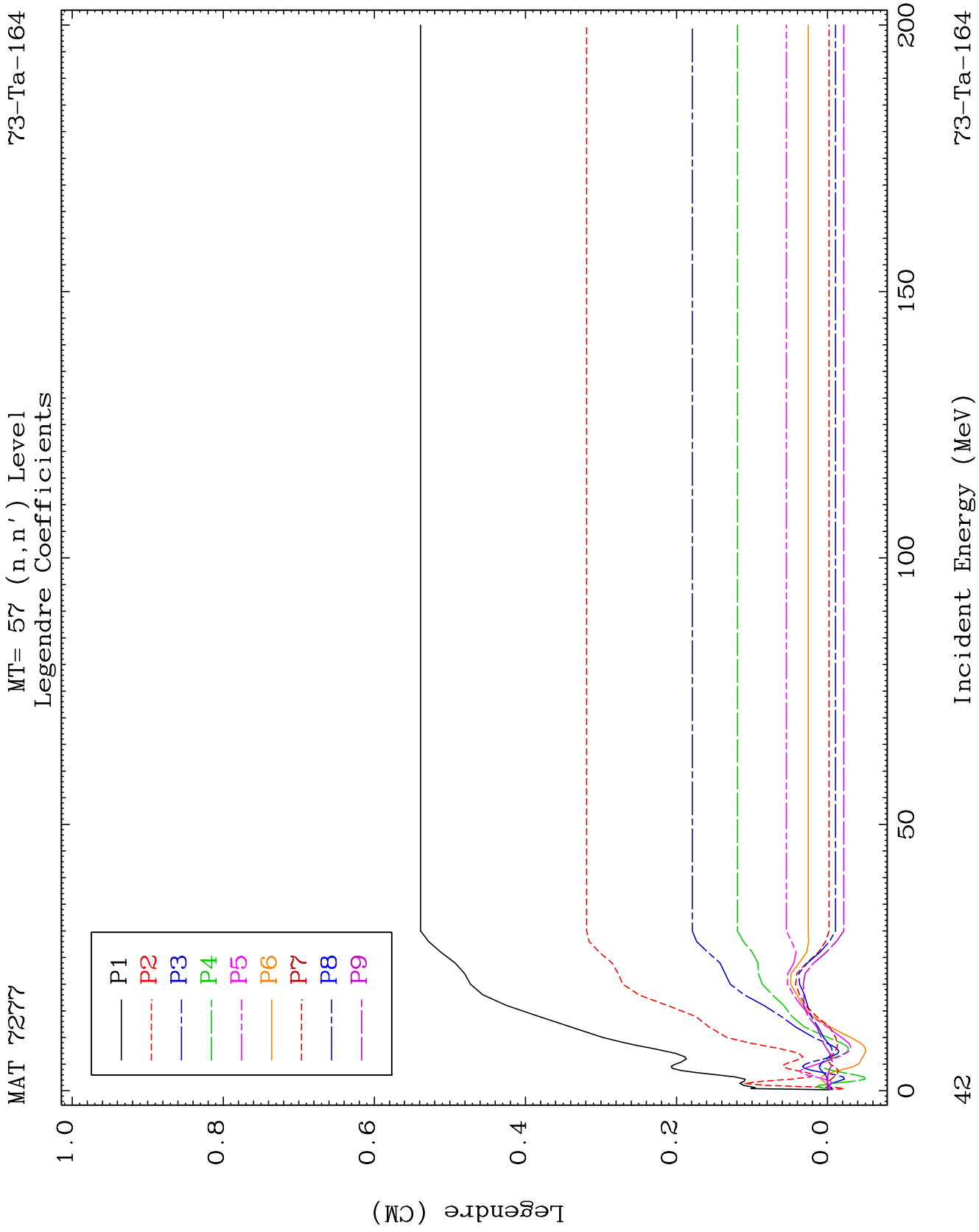
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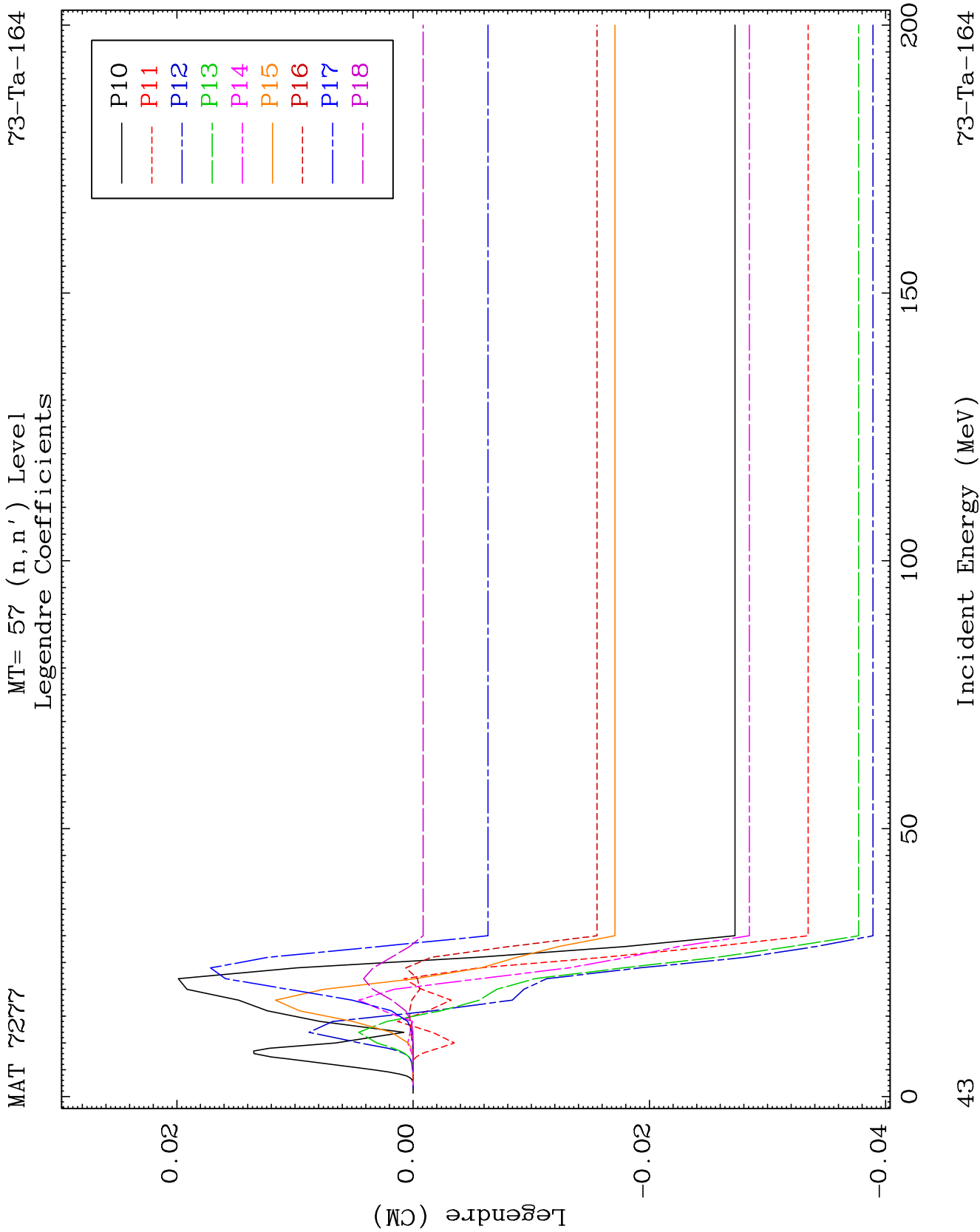
MT= 56 (n, n') Level
Legendre Coefficients

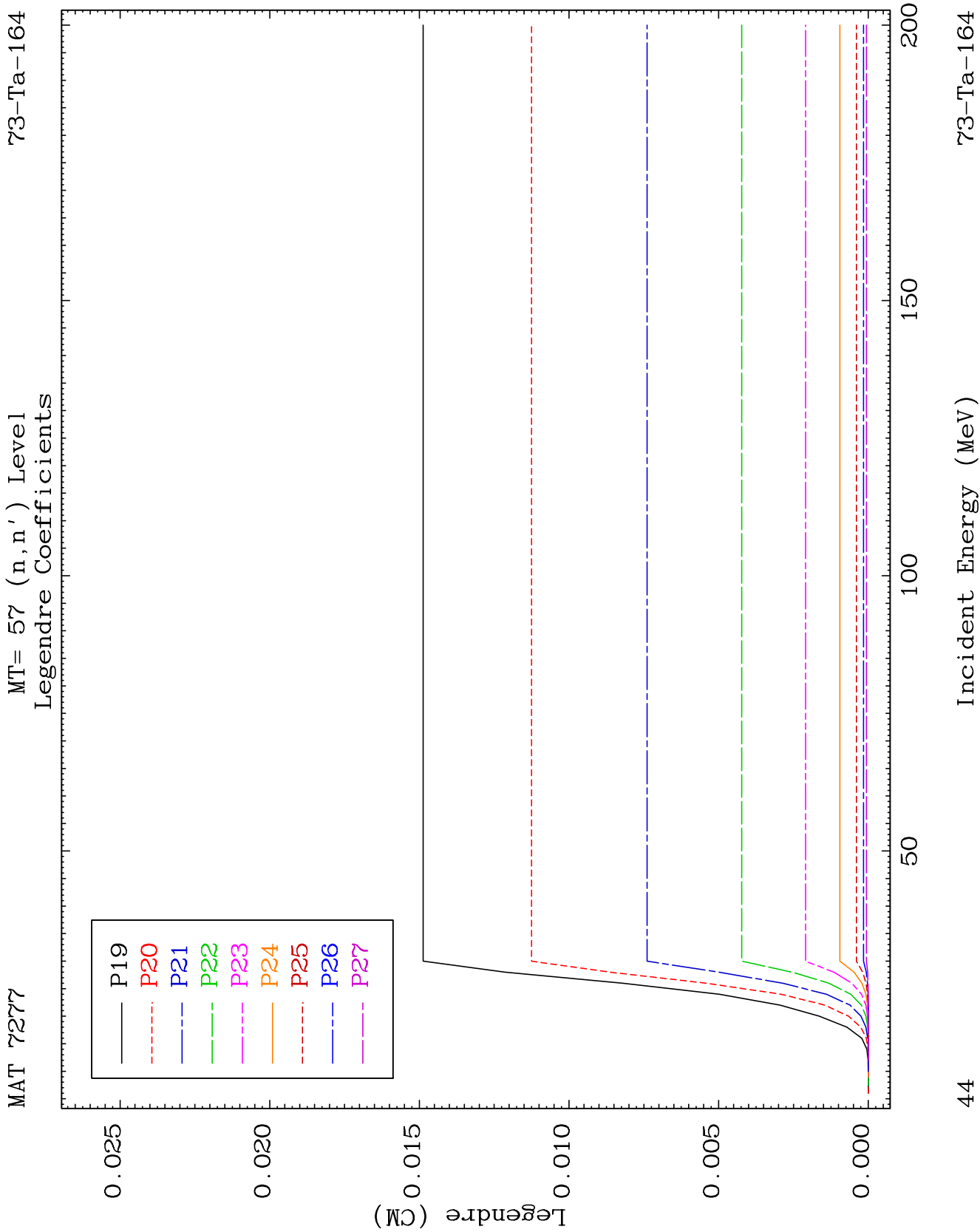
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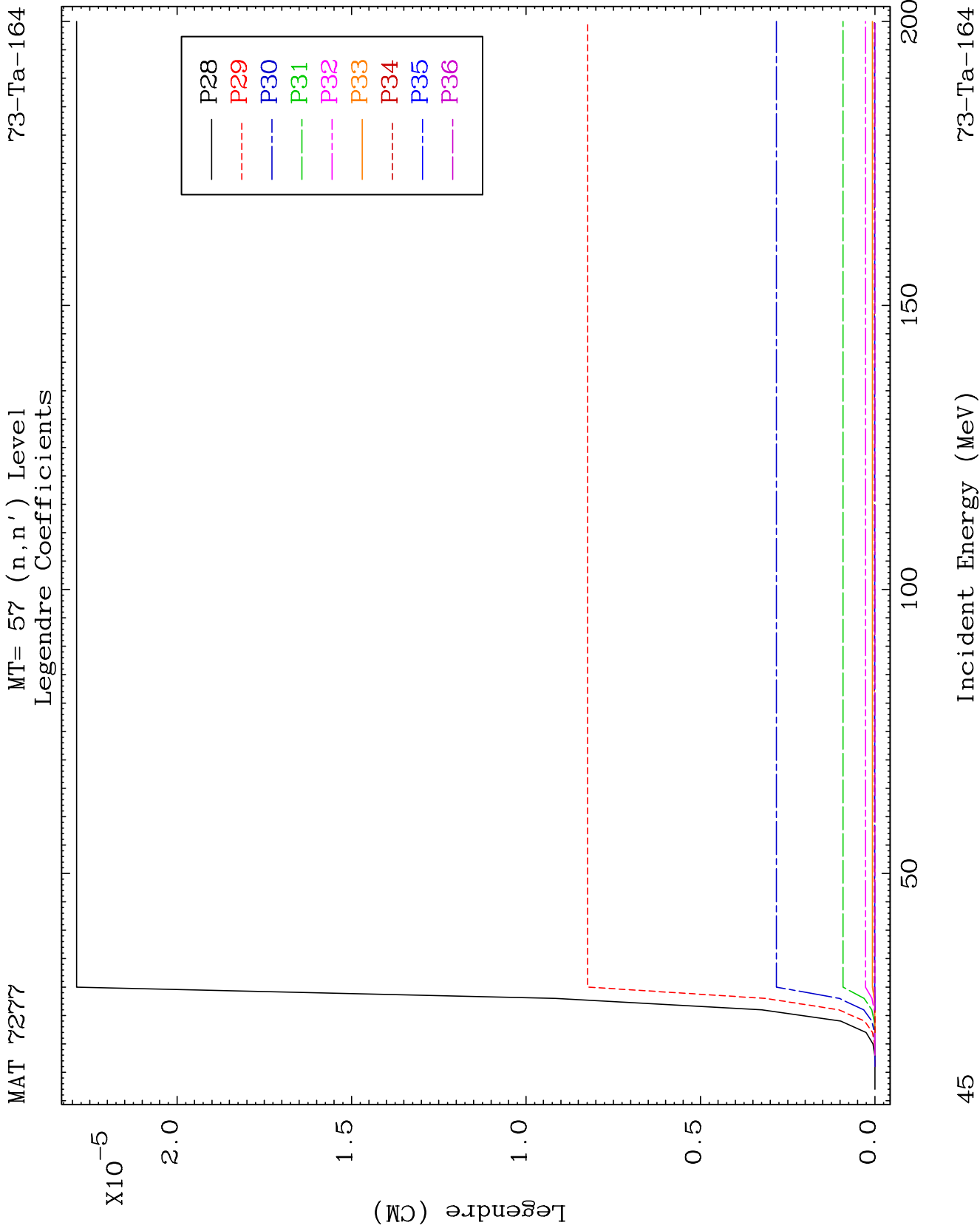








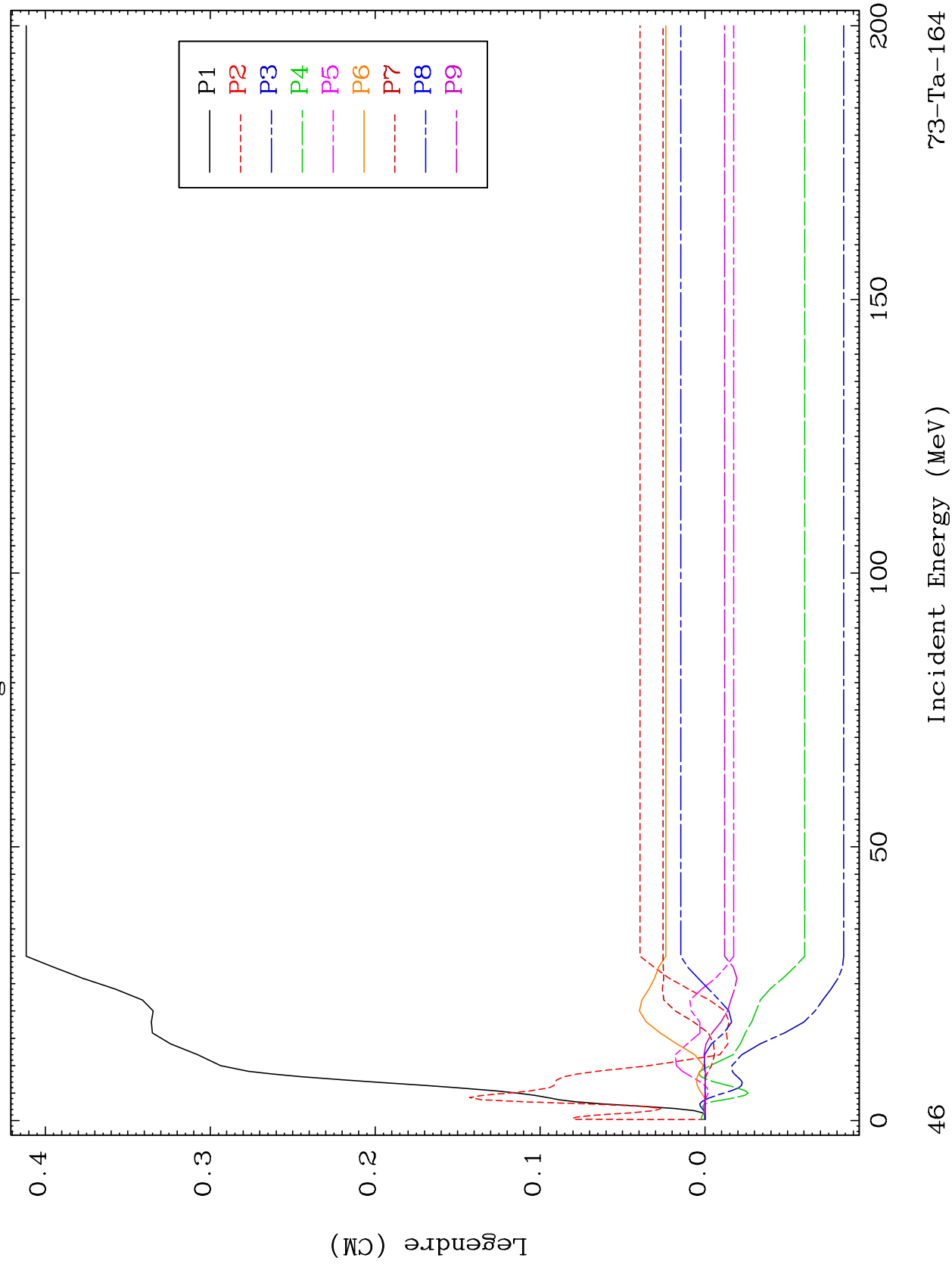




MAT 7277

MT= 58 (n, n') Level
Legendre Coefficients

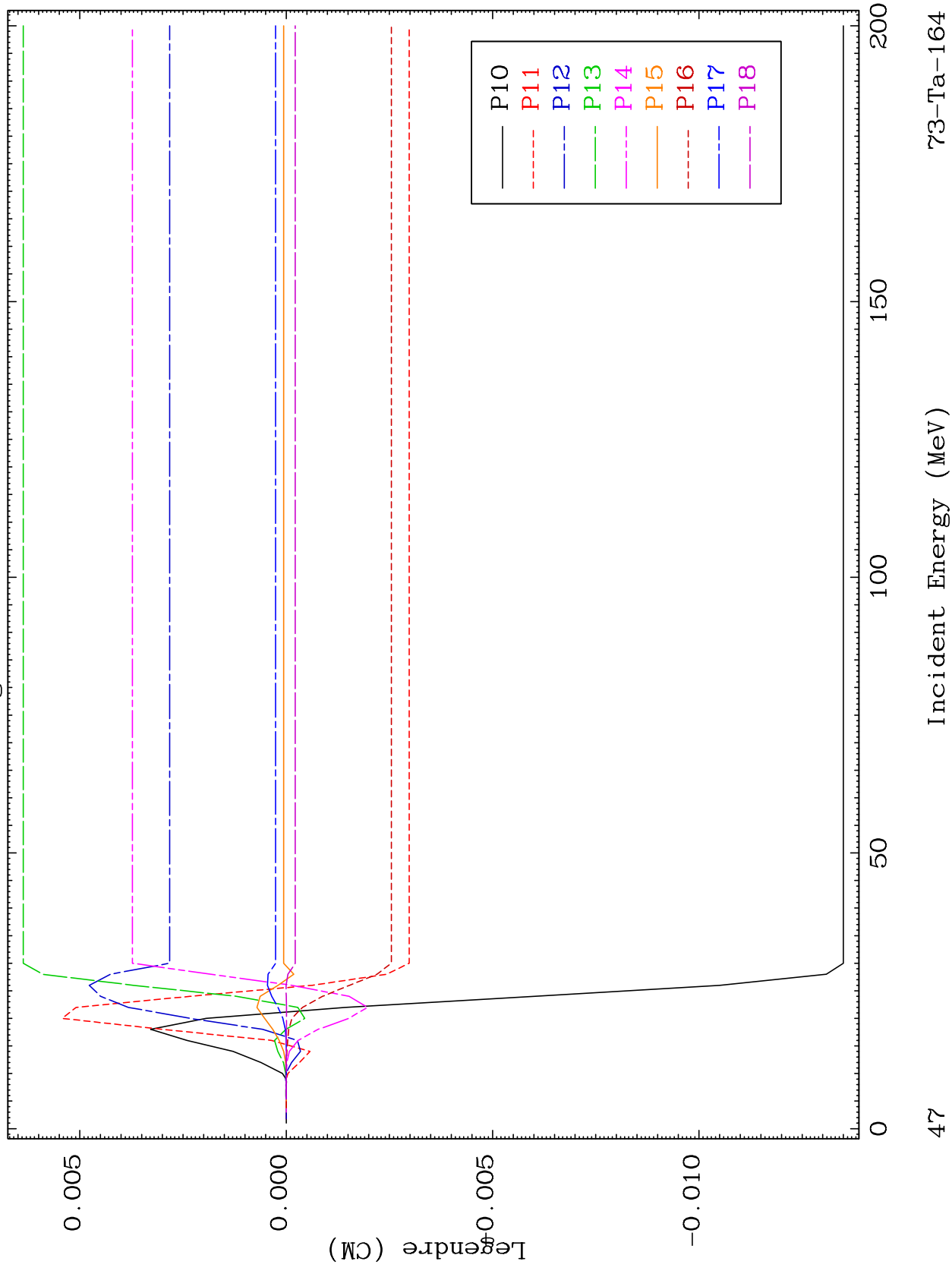
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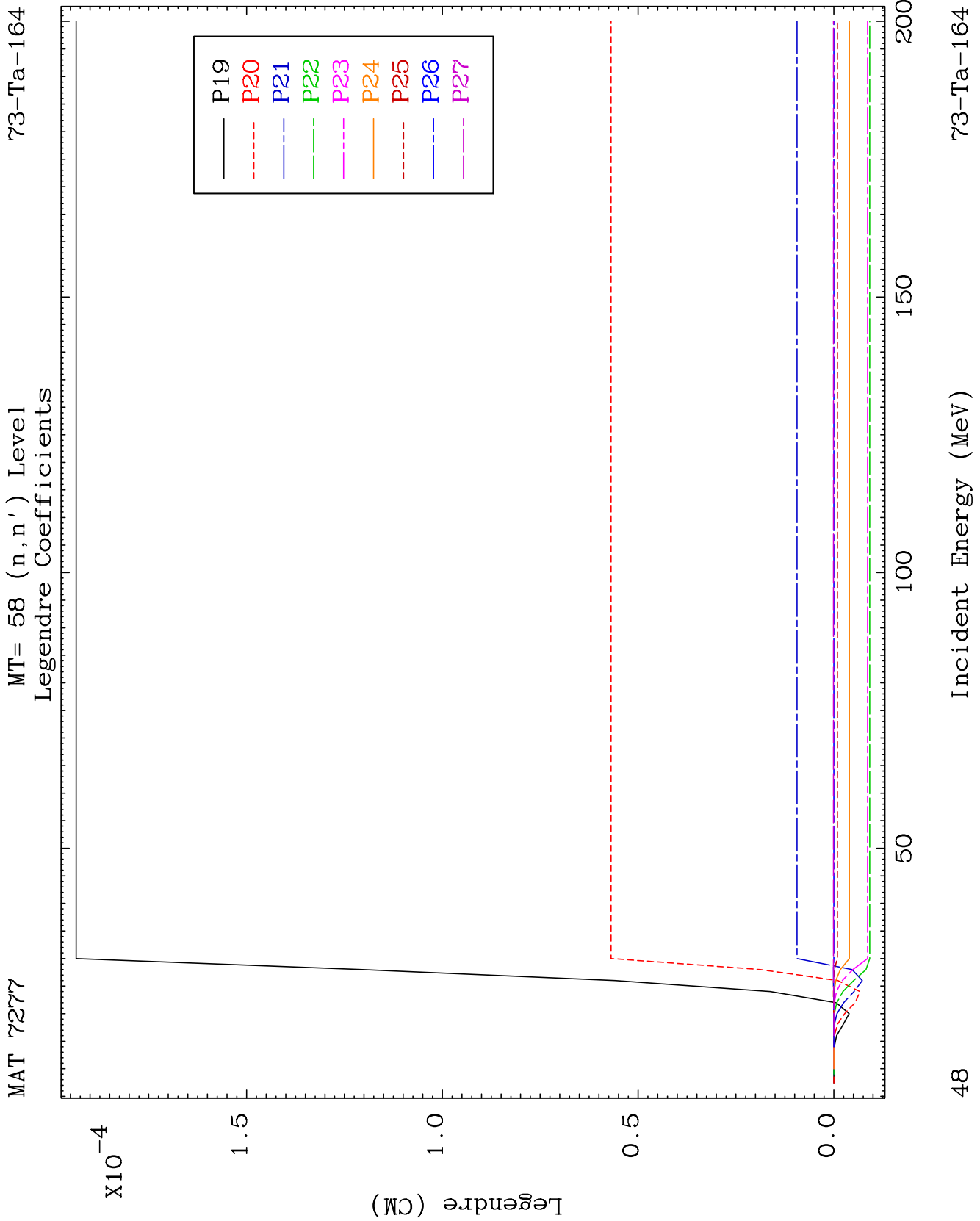


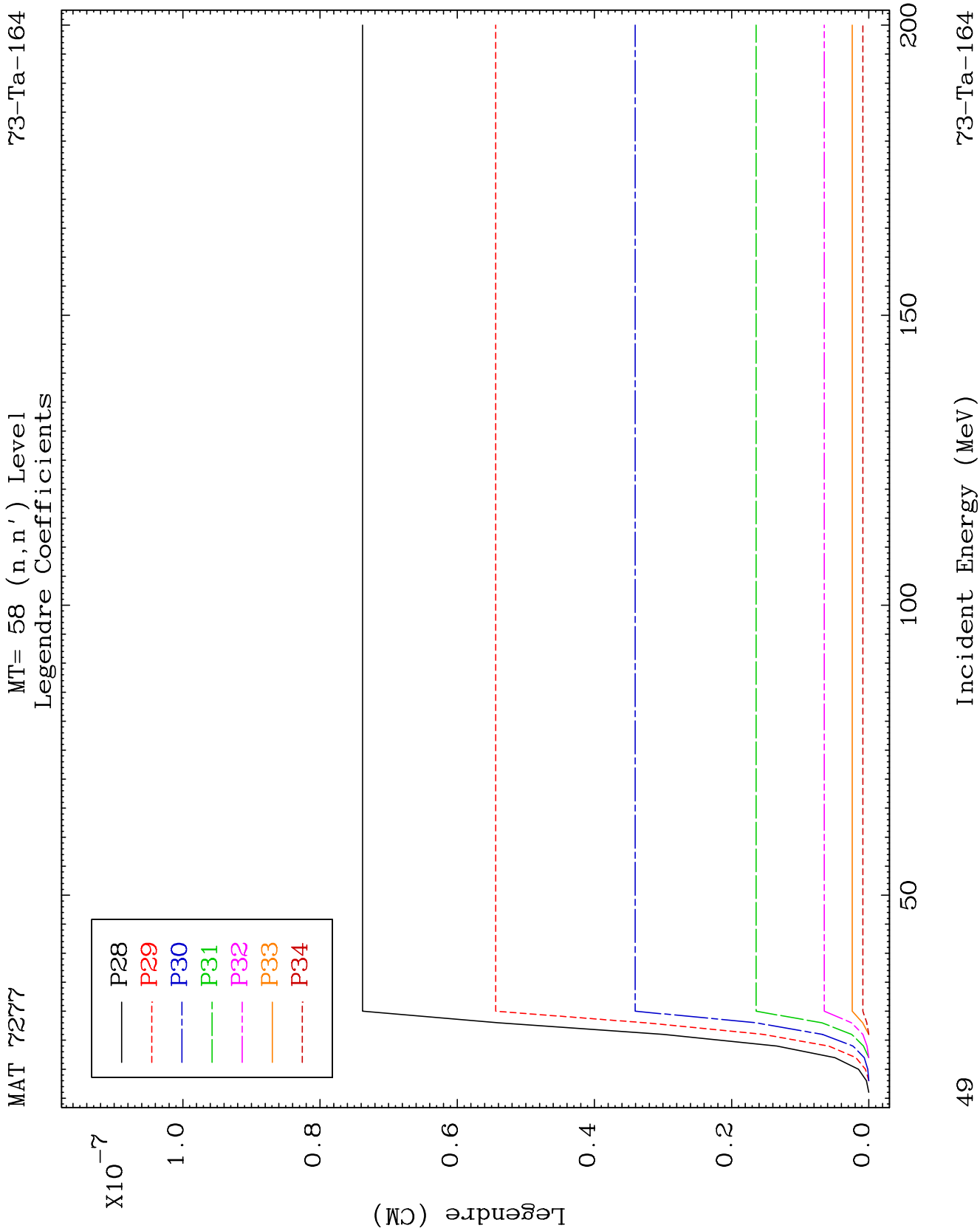
MAT 7277

MT= 58 (n, n') Level
Legendre Coefficients

73-Ta-164



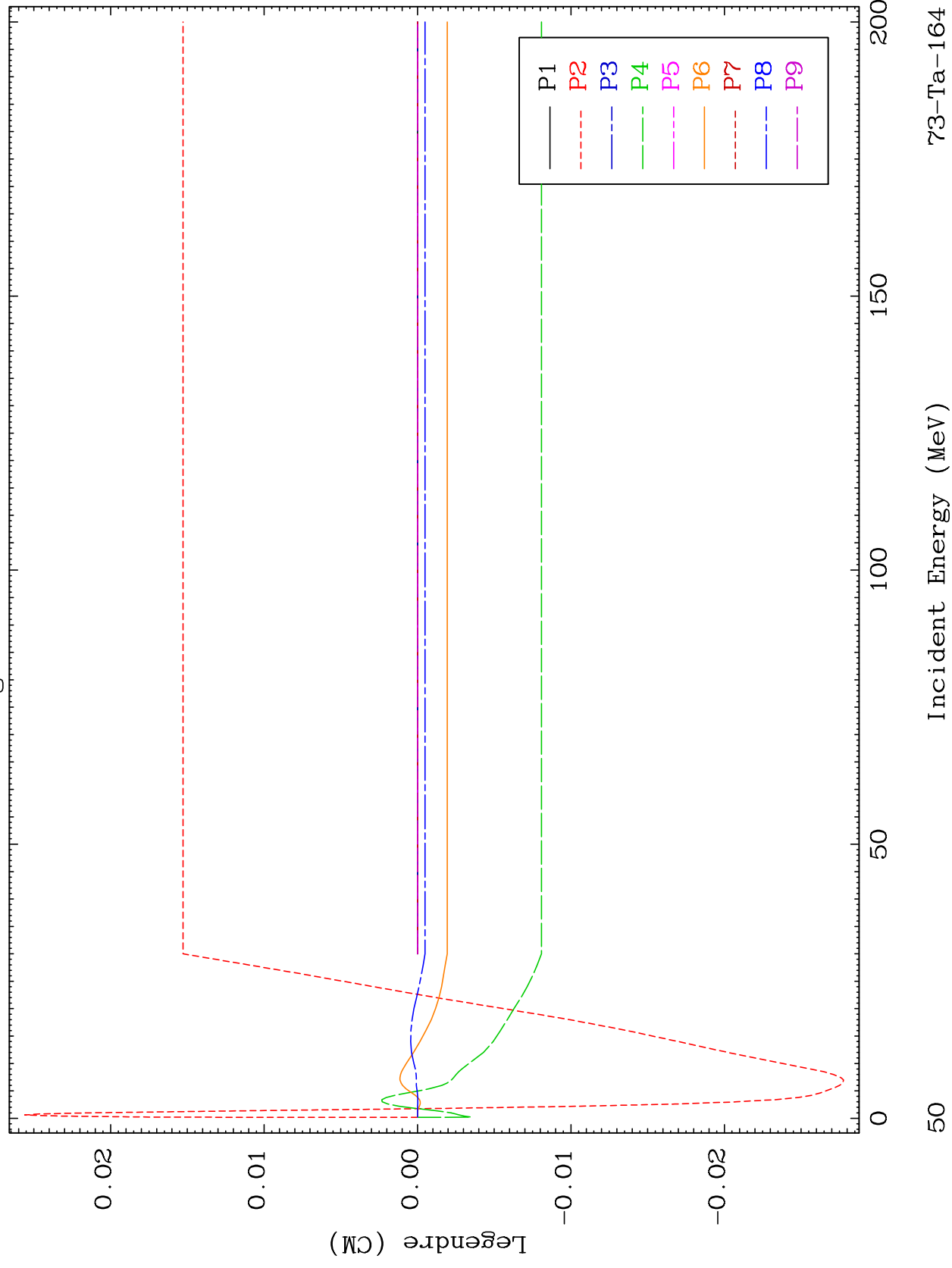


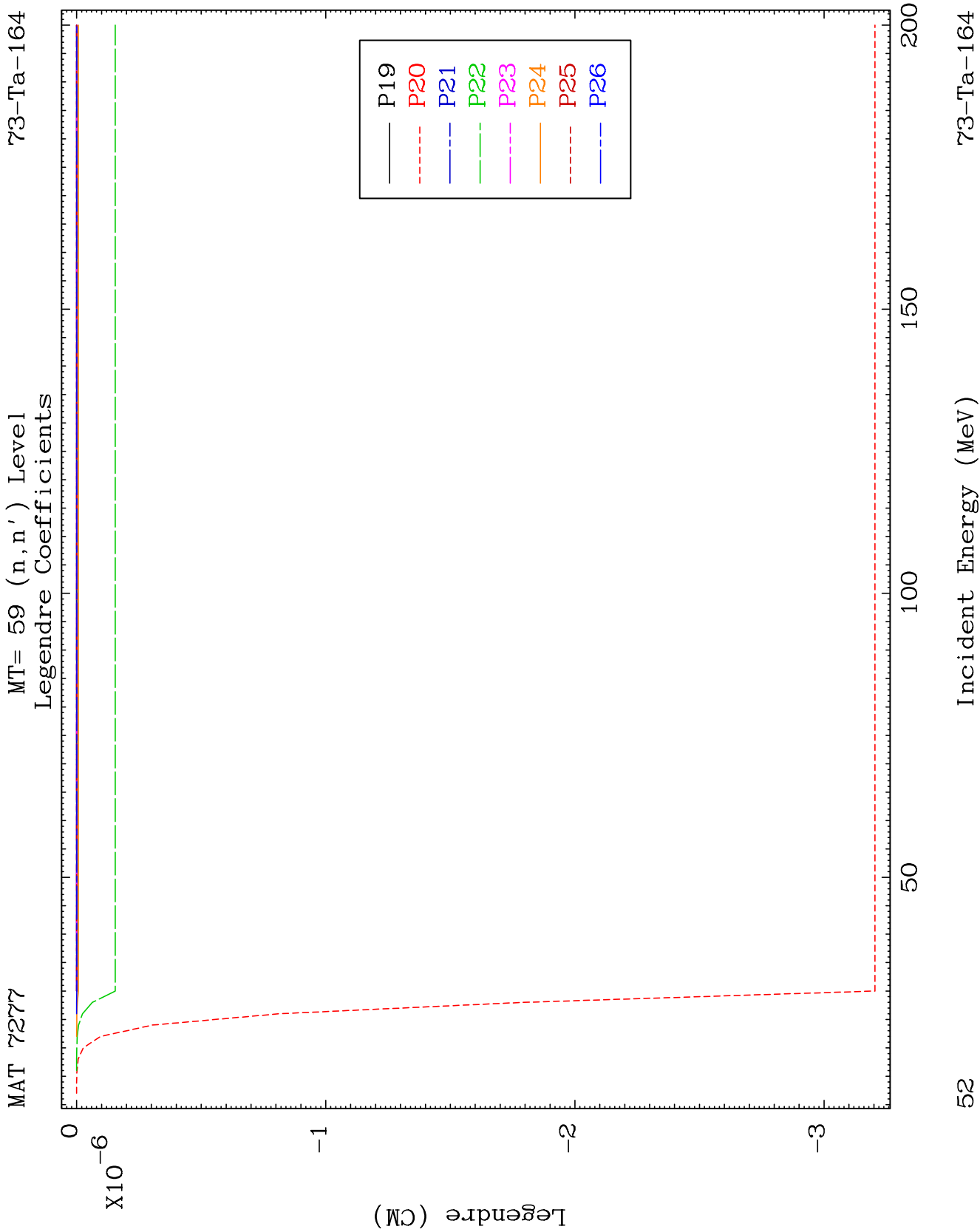


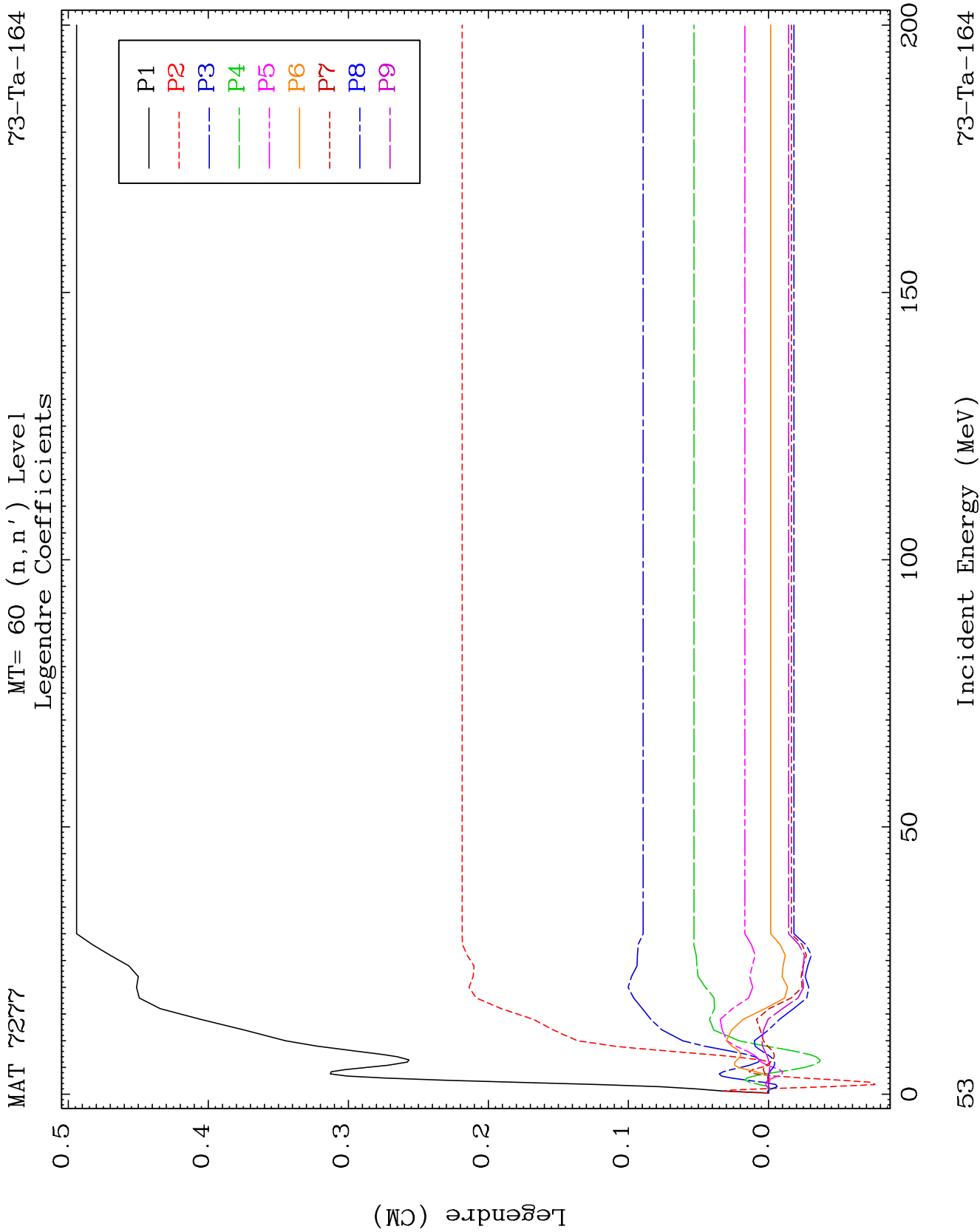
MAT 7277

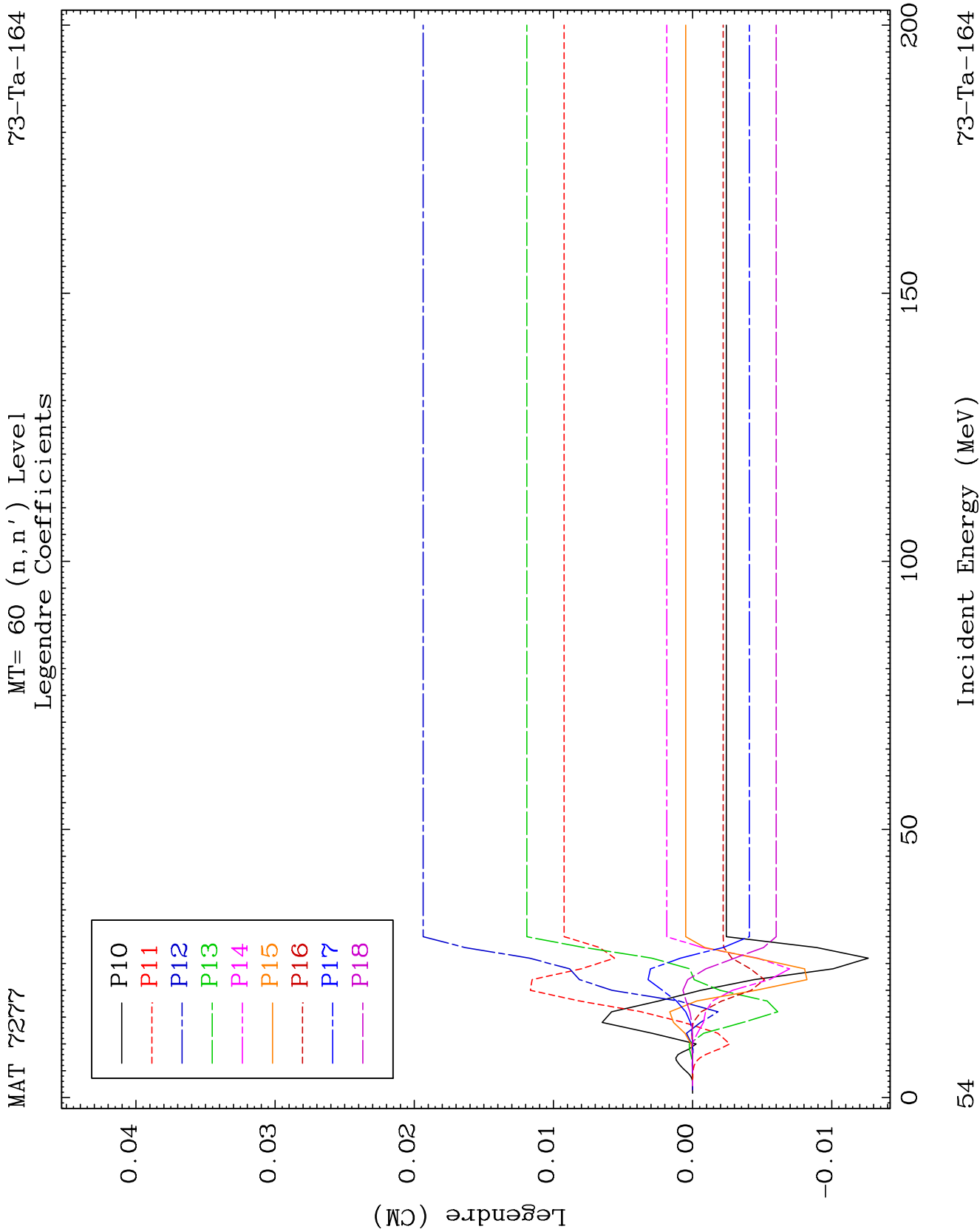
MT= 59 (n, n') Level
Legendre Coefficients

73-Ta-164





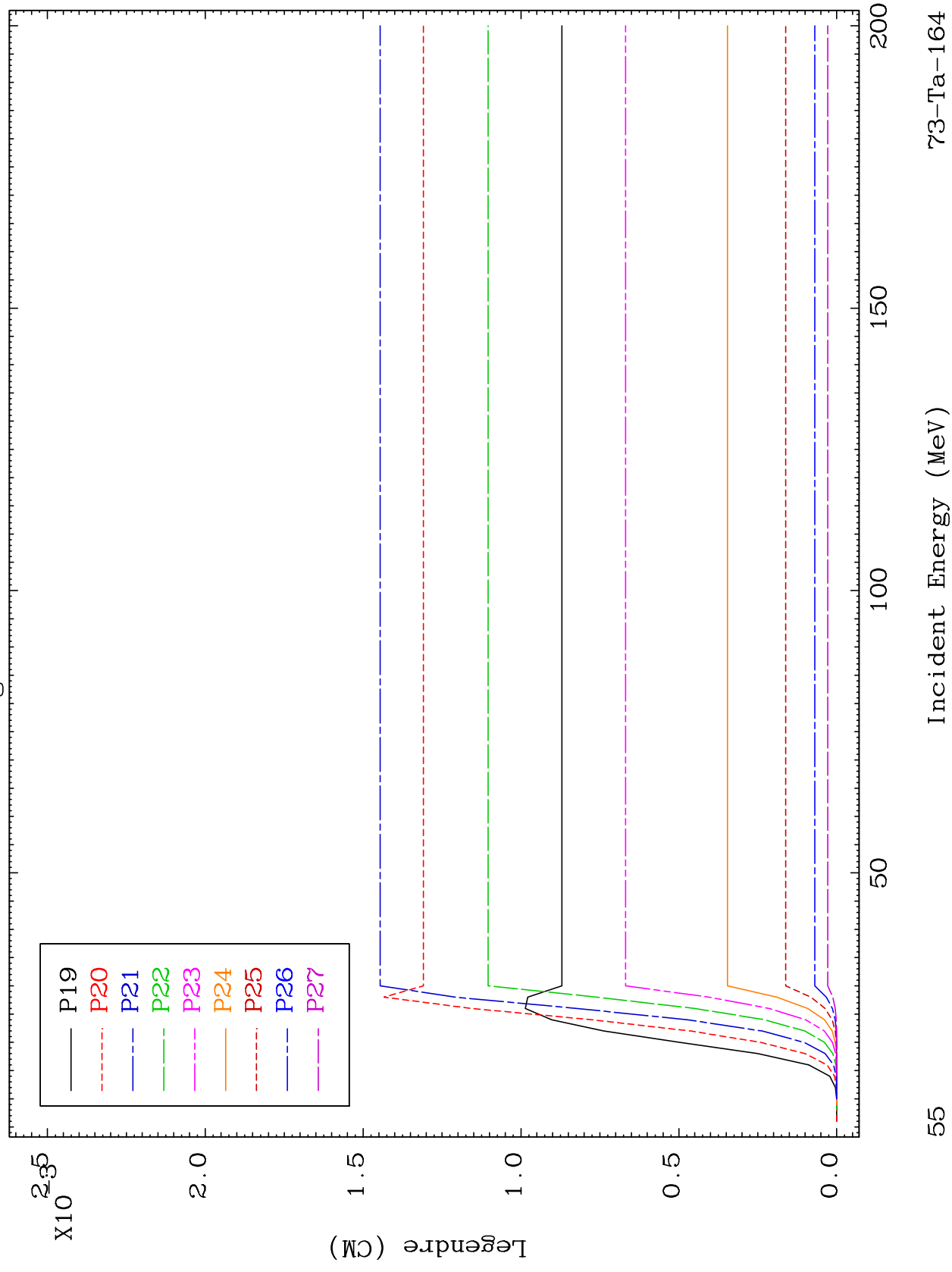




MAT 7277

MT= 60 (n, n') Level
Legendre Coefficients

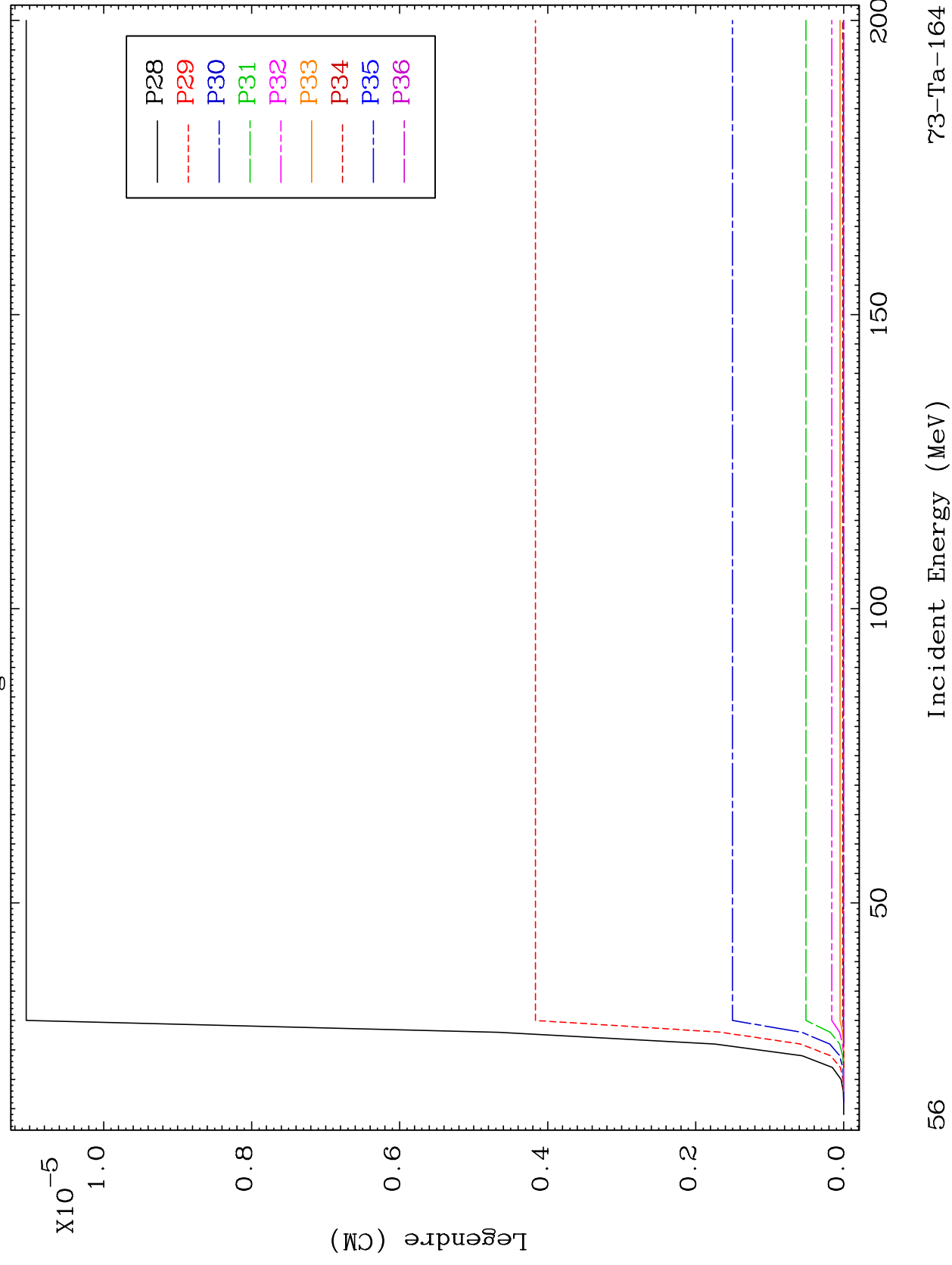
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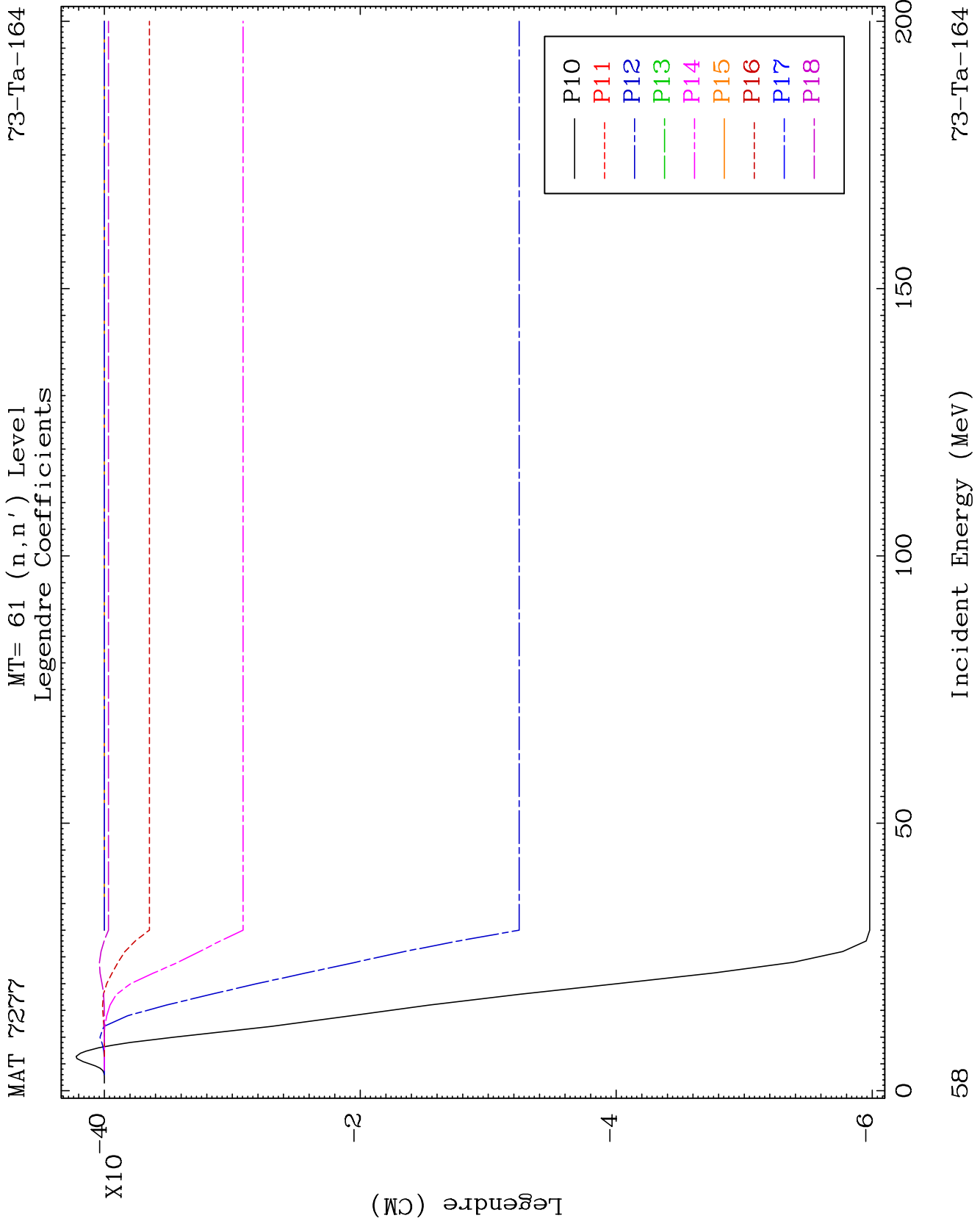


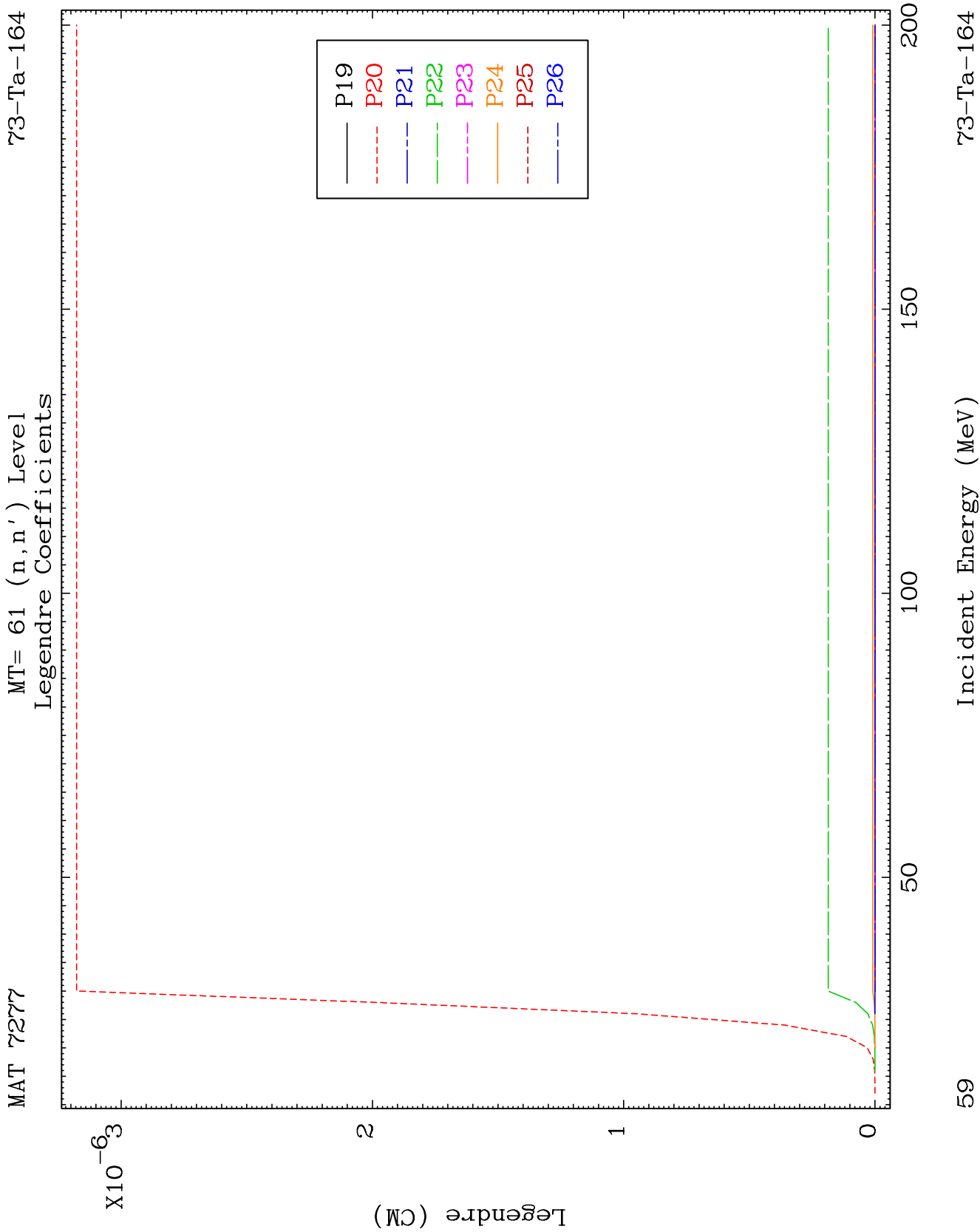
MAT 7277

MT= 60 (n, n') Level
Legendre Coefficients

73-Ta-164



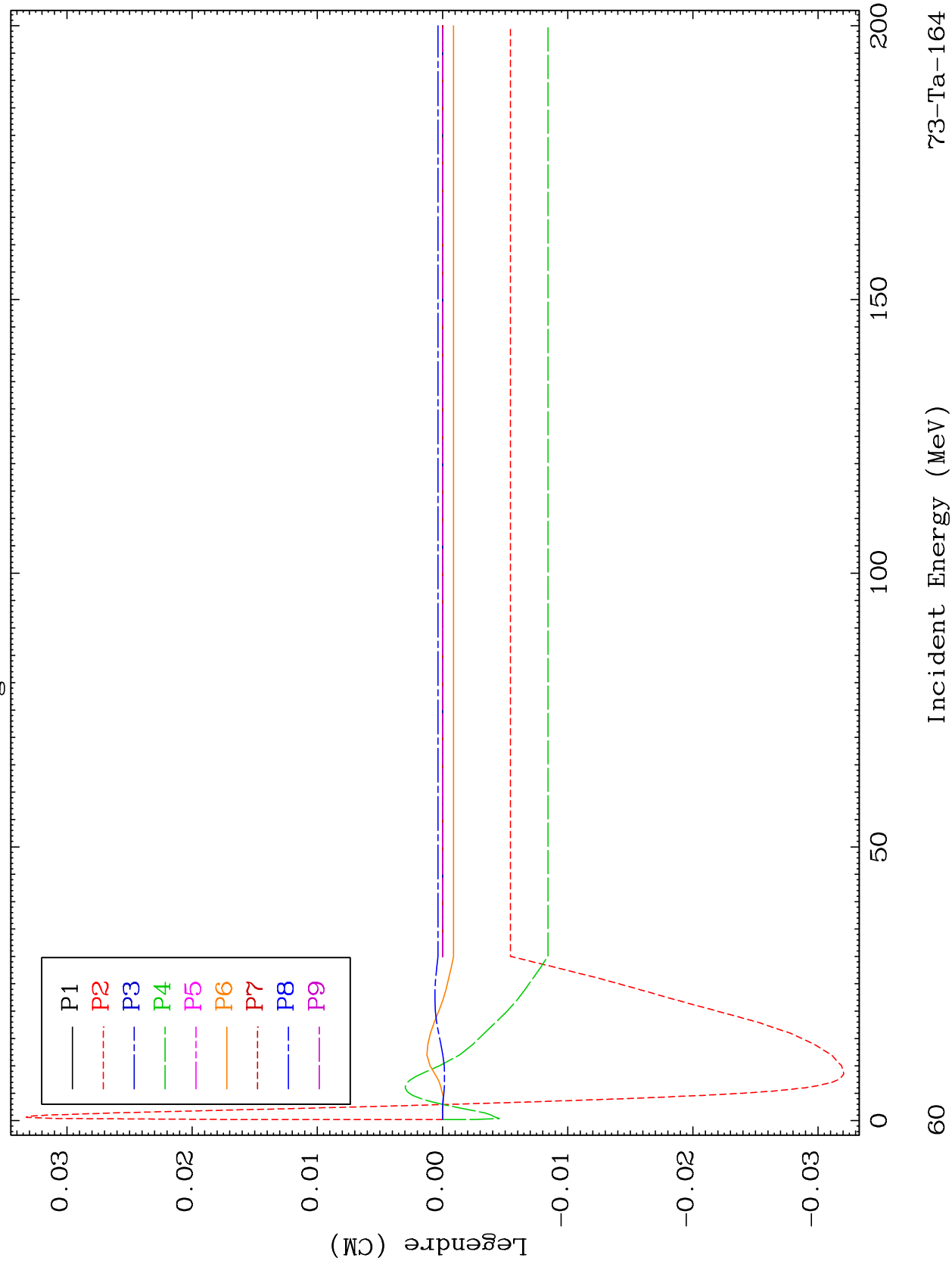


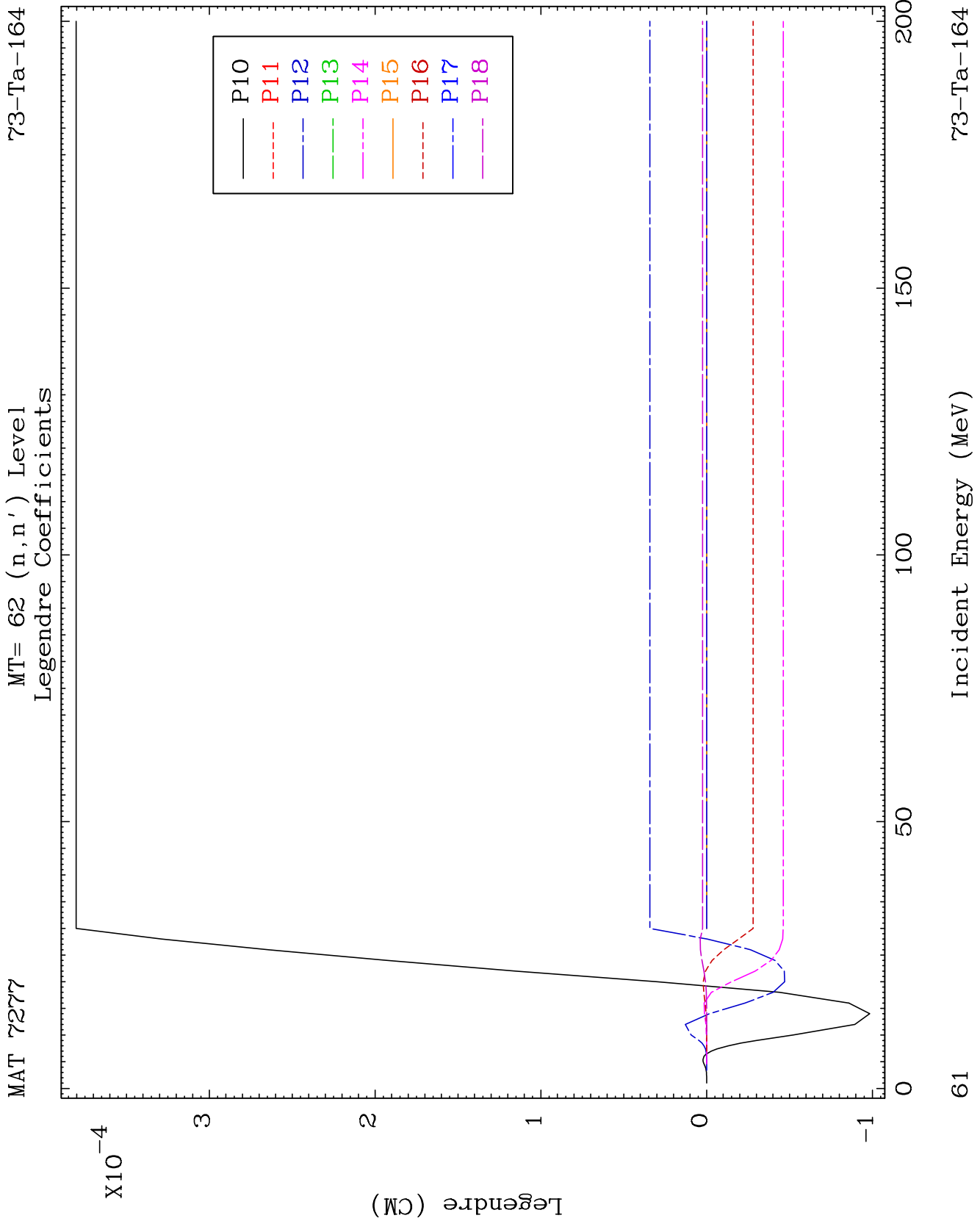


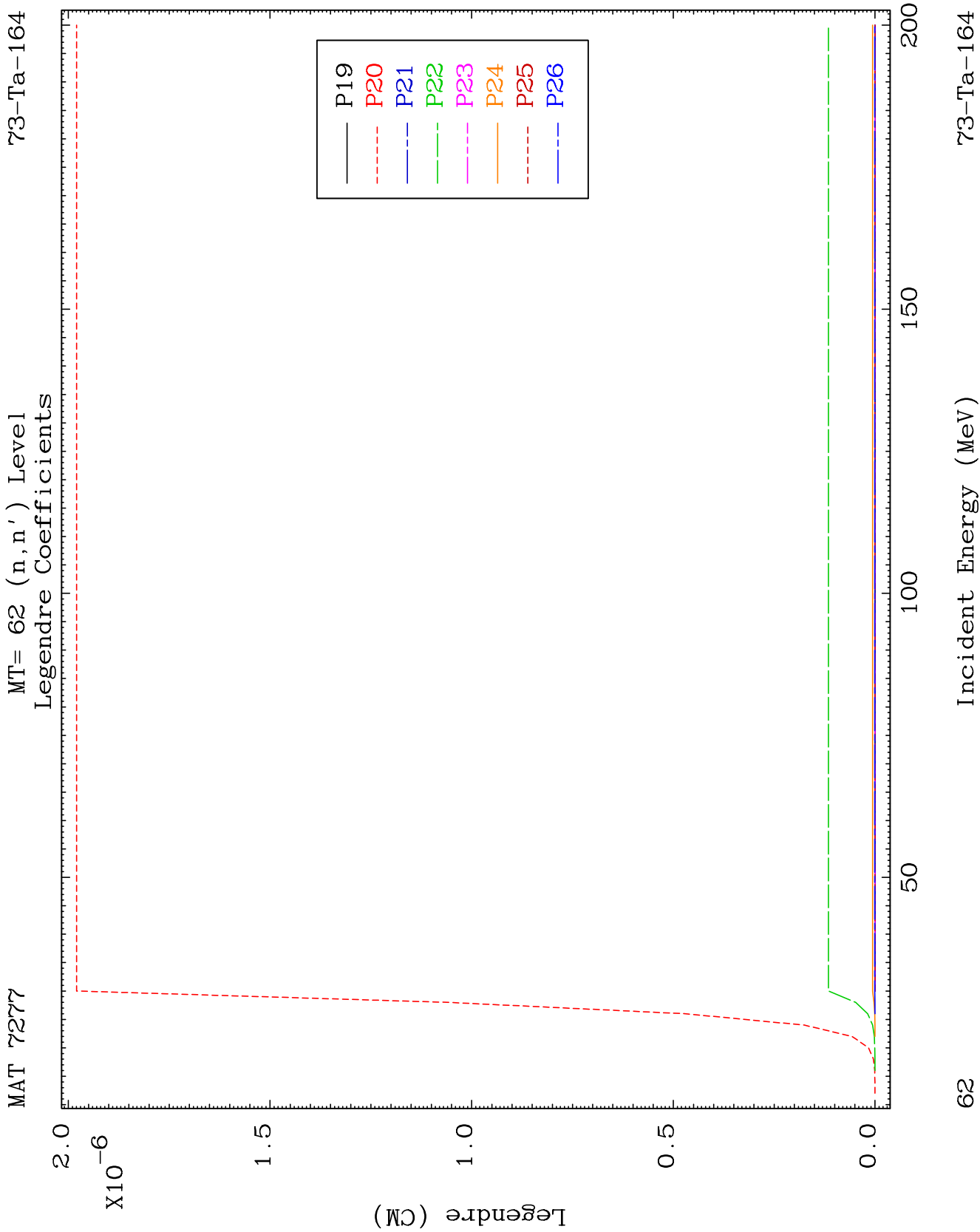
MAT 7277

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Legendre Coefficients

73-Ta-164



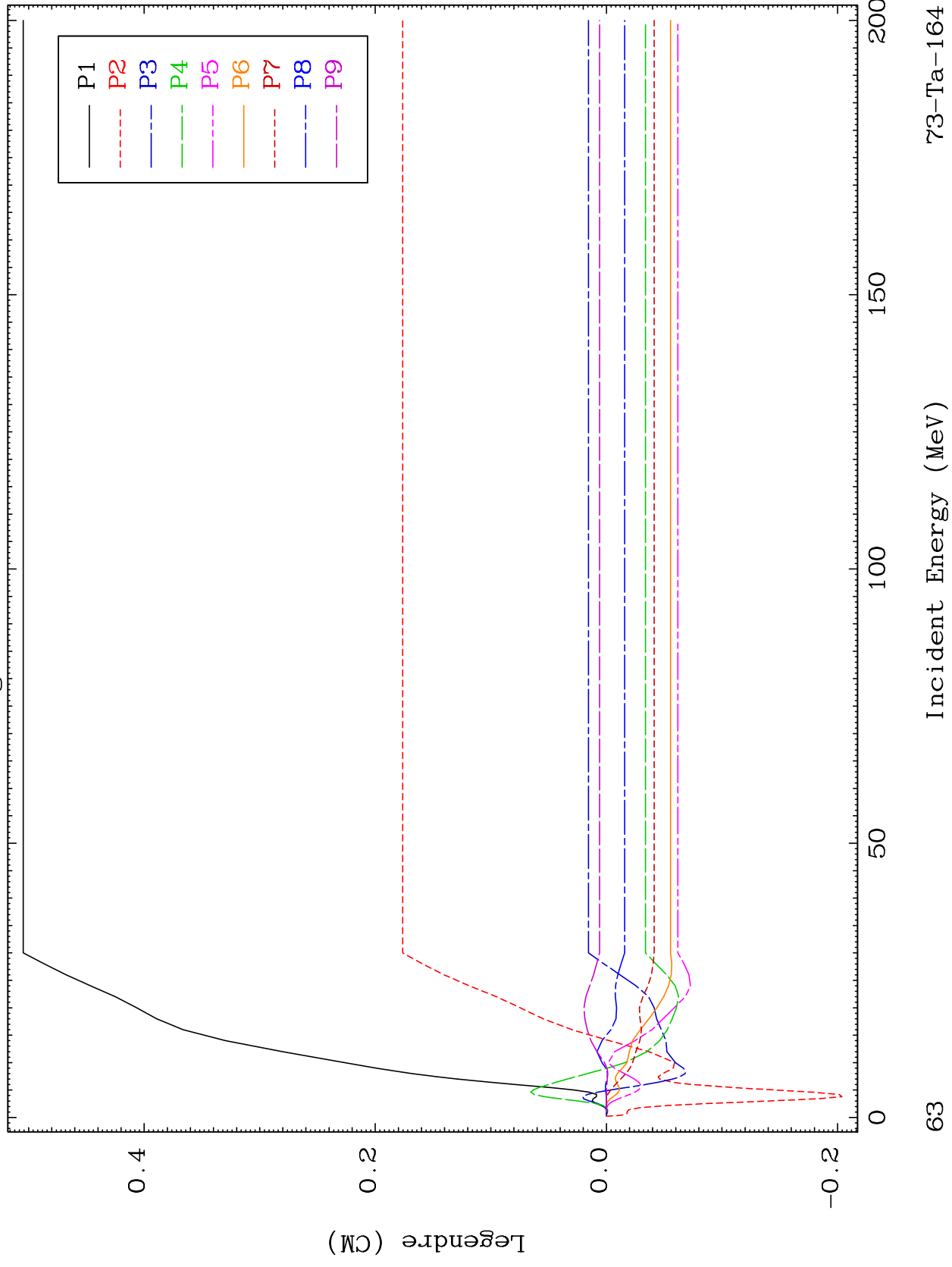


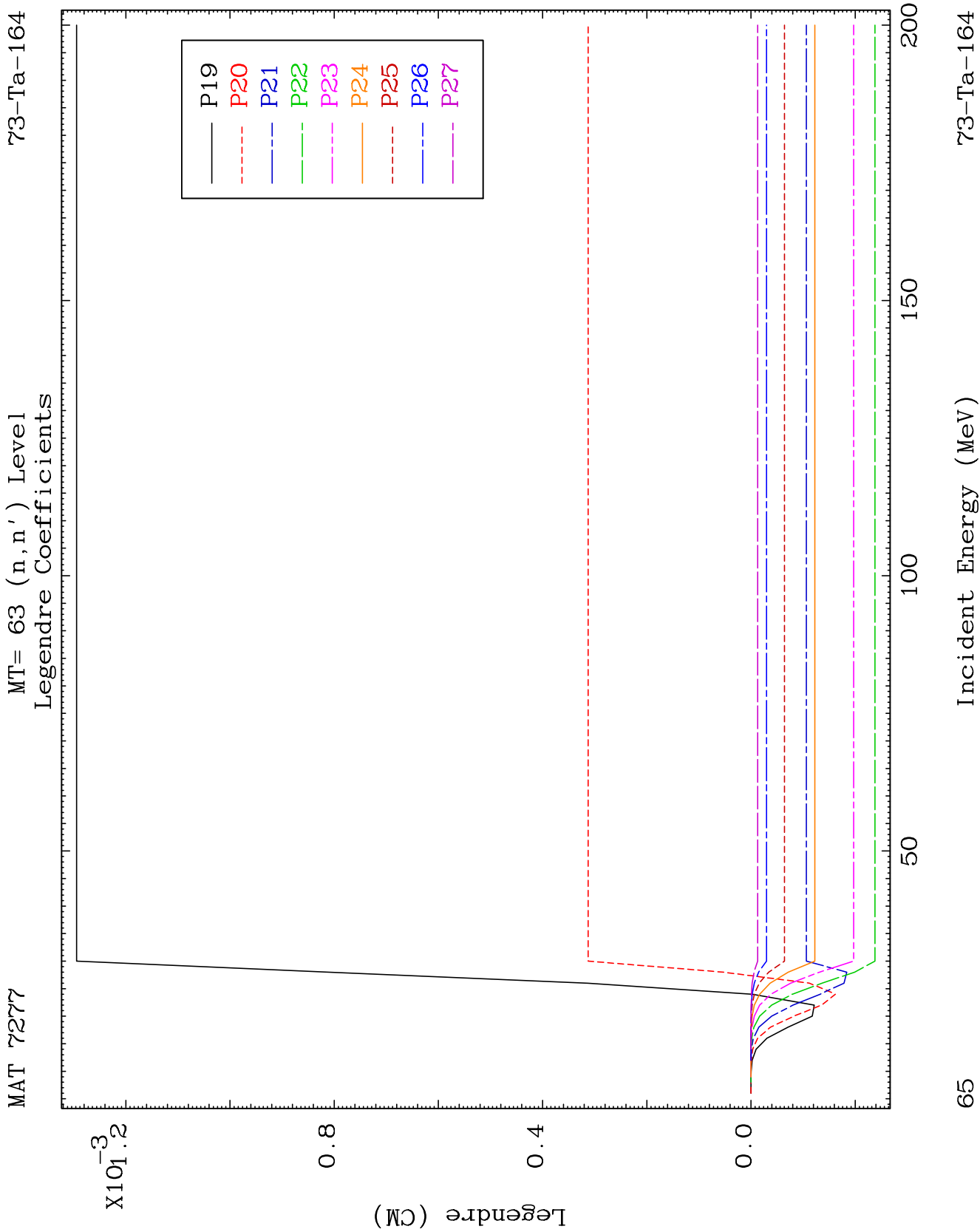


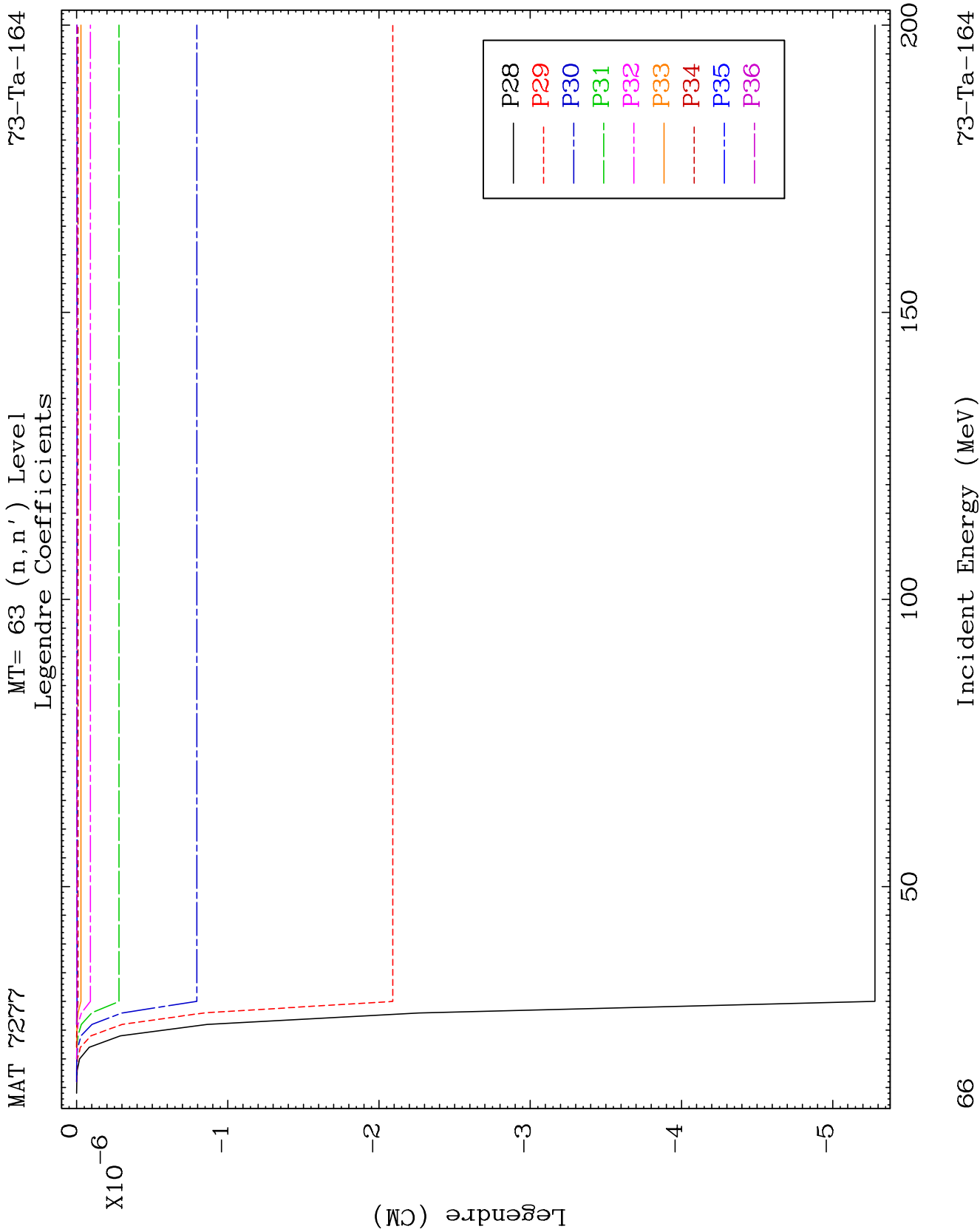
MAT 7277

MT= 63 (n, n') Level
Legendre Coefficients

73-Ta-164



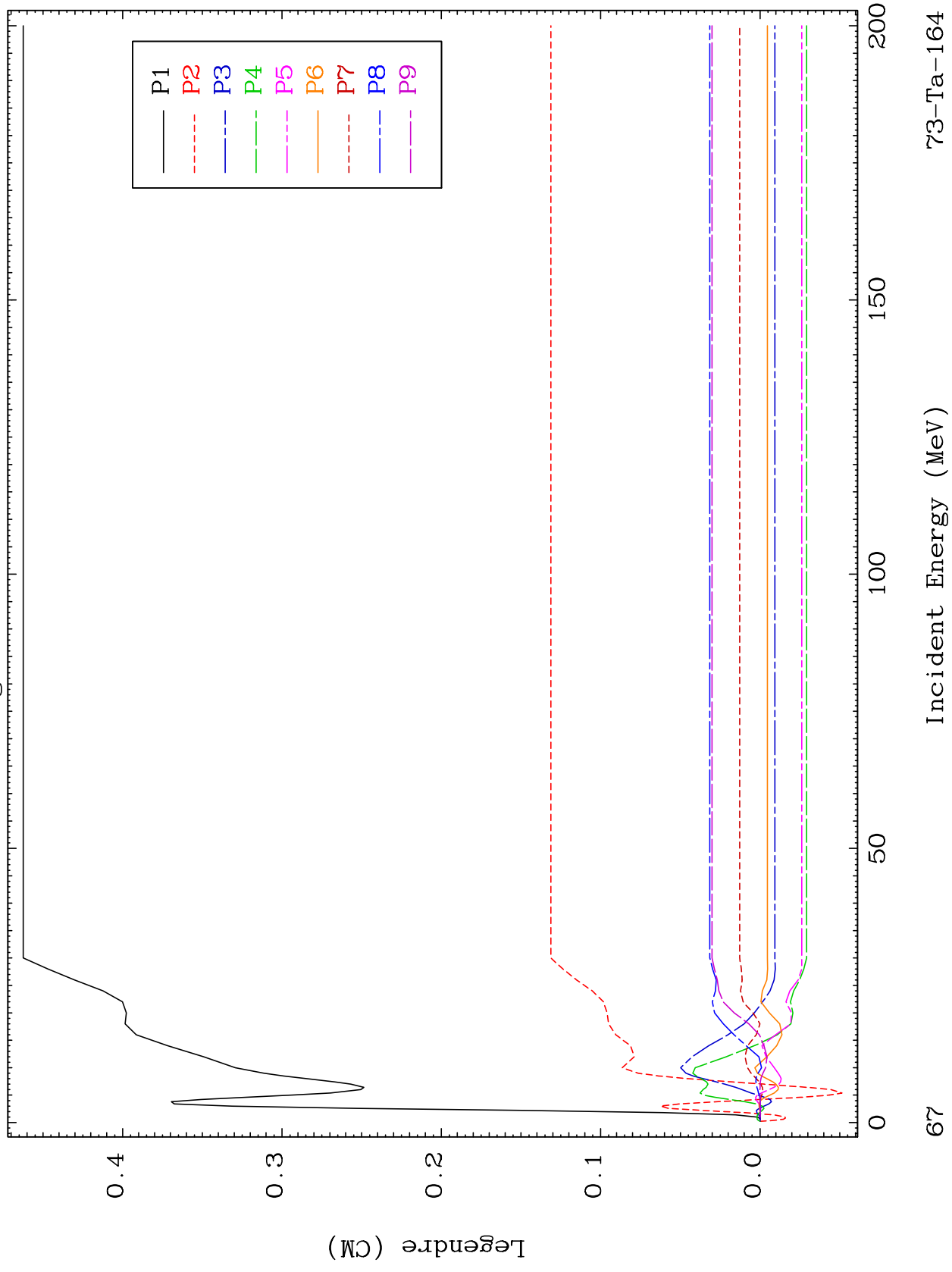


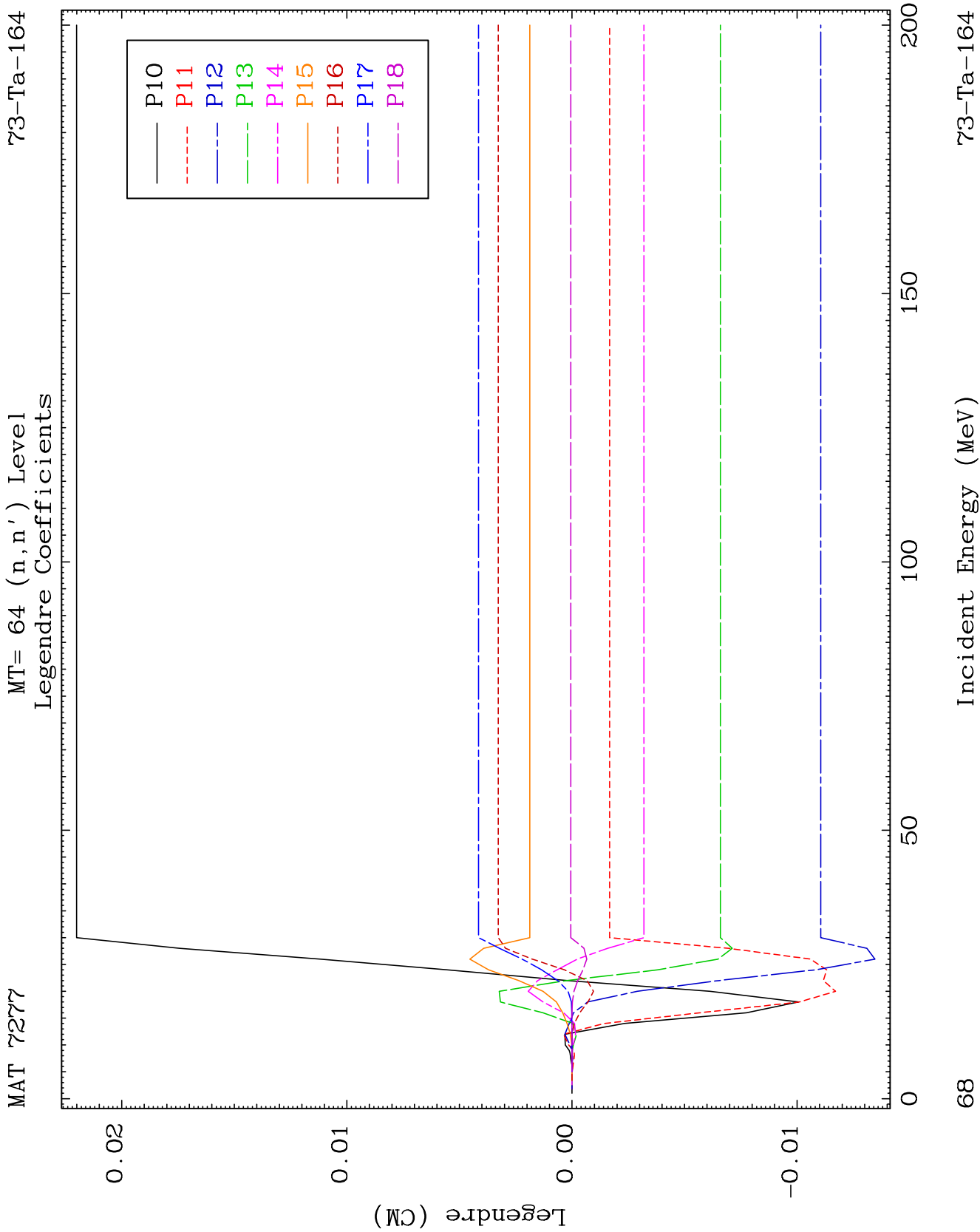


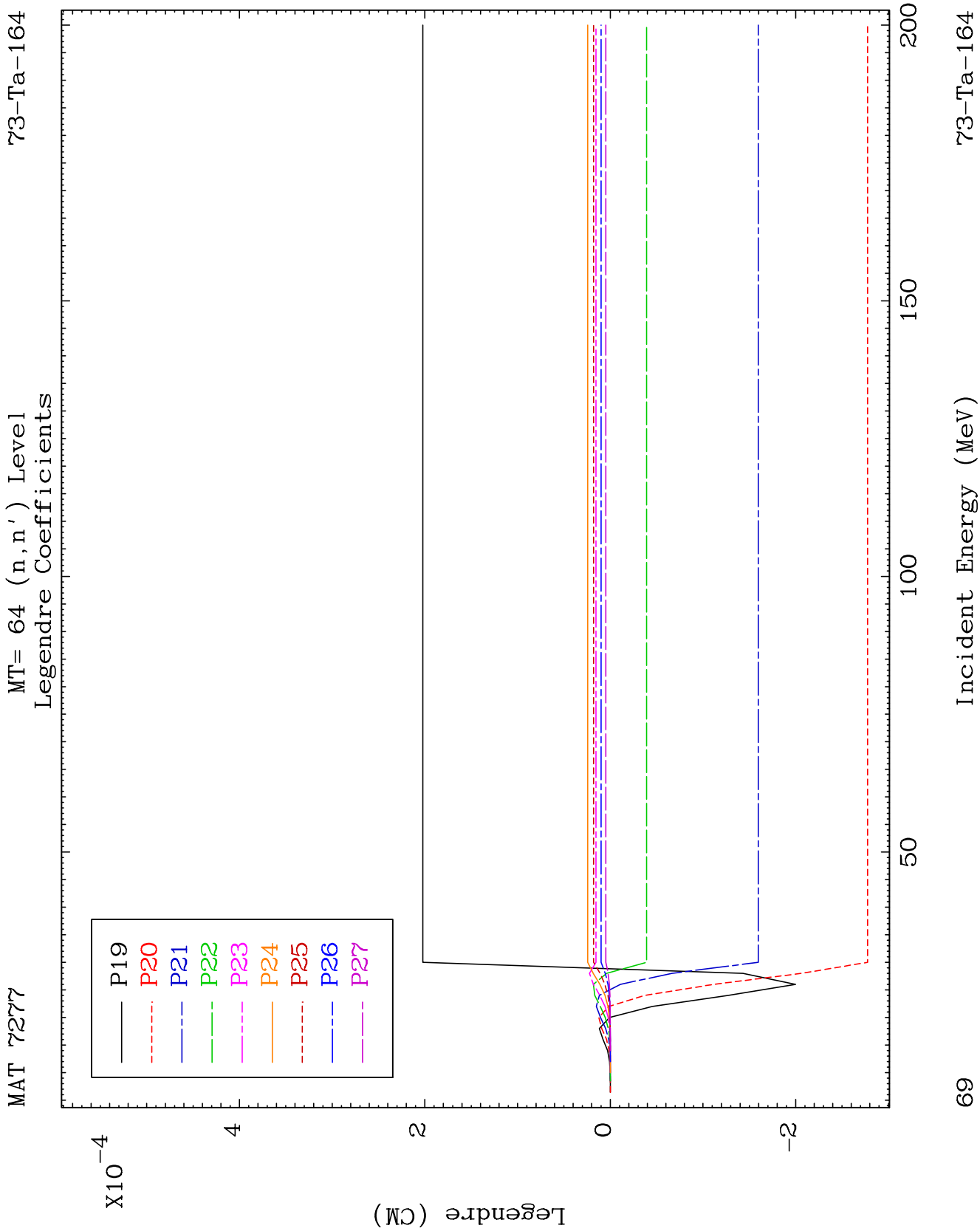
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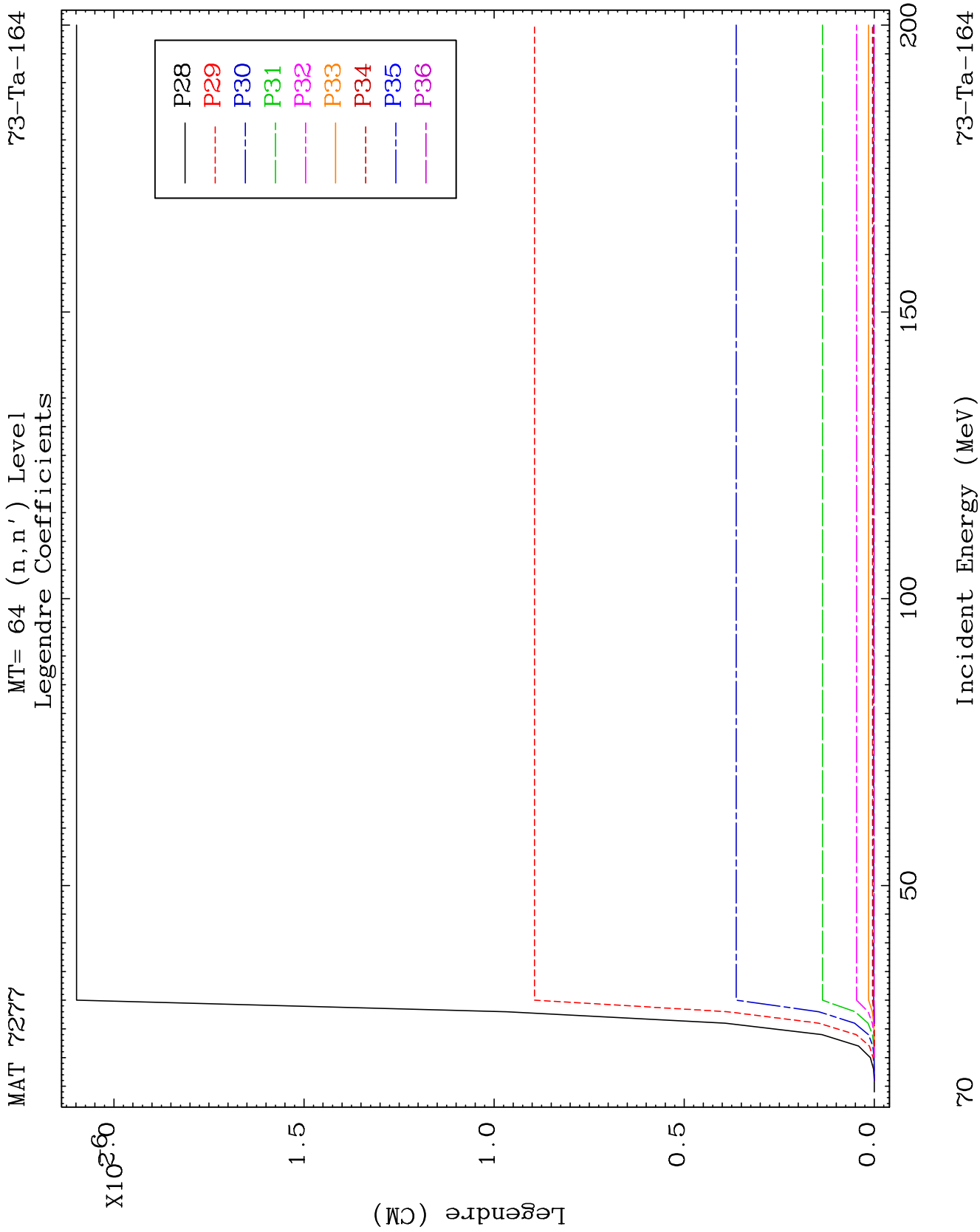
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Legendre Coefficients

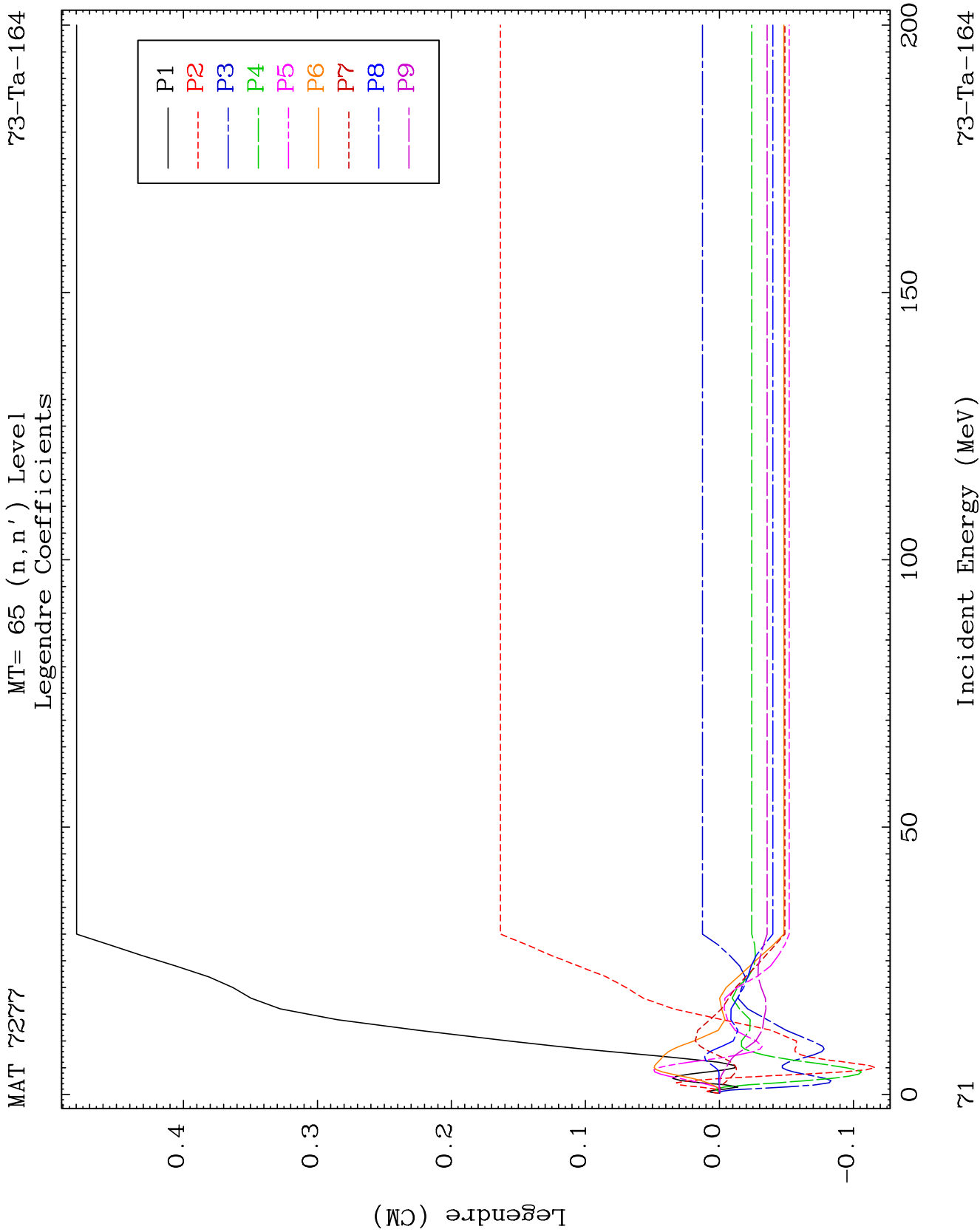
73-Ta-164

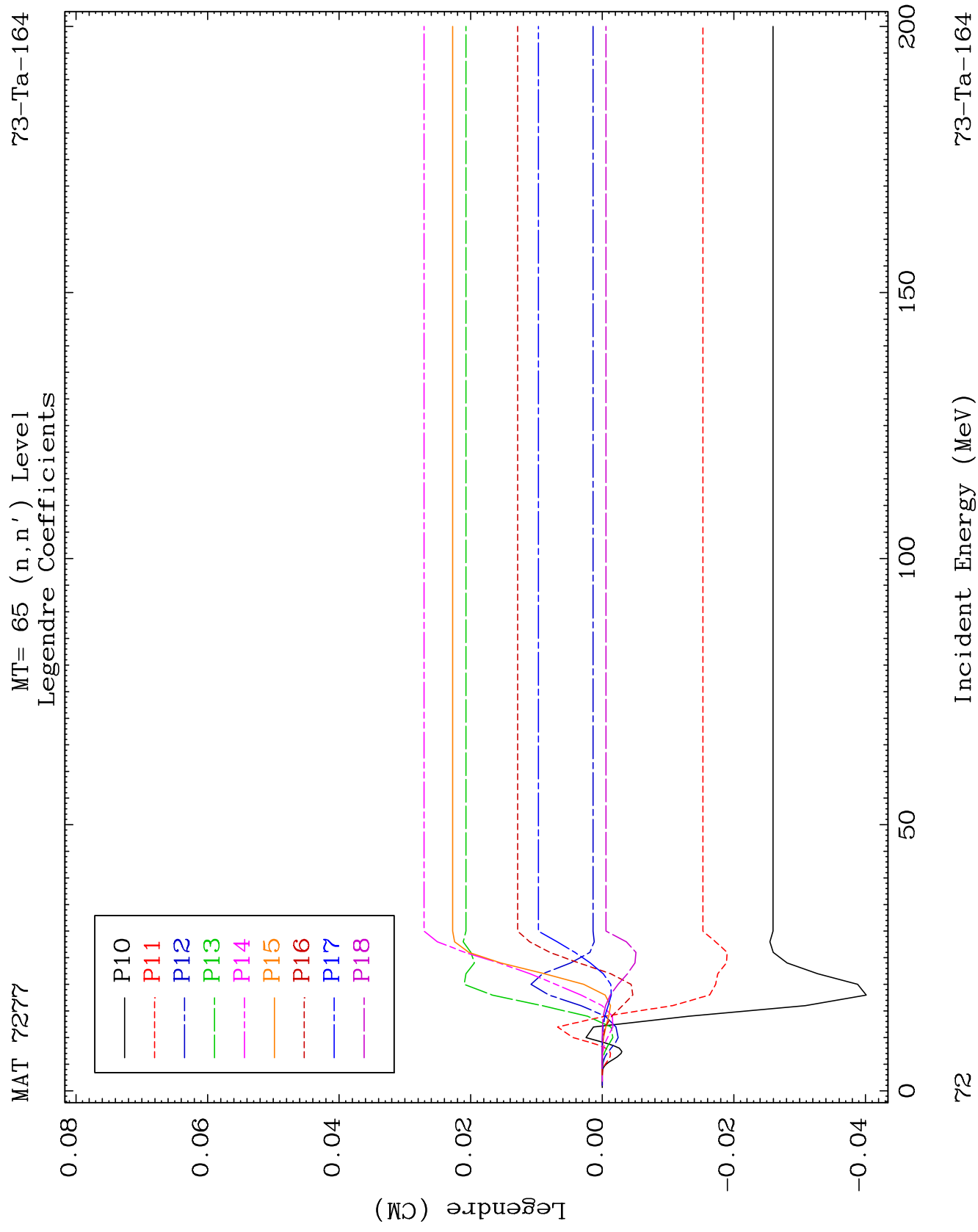


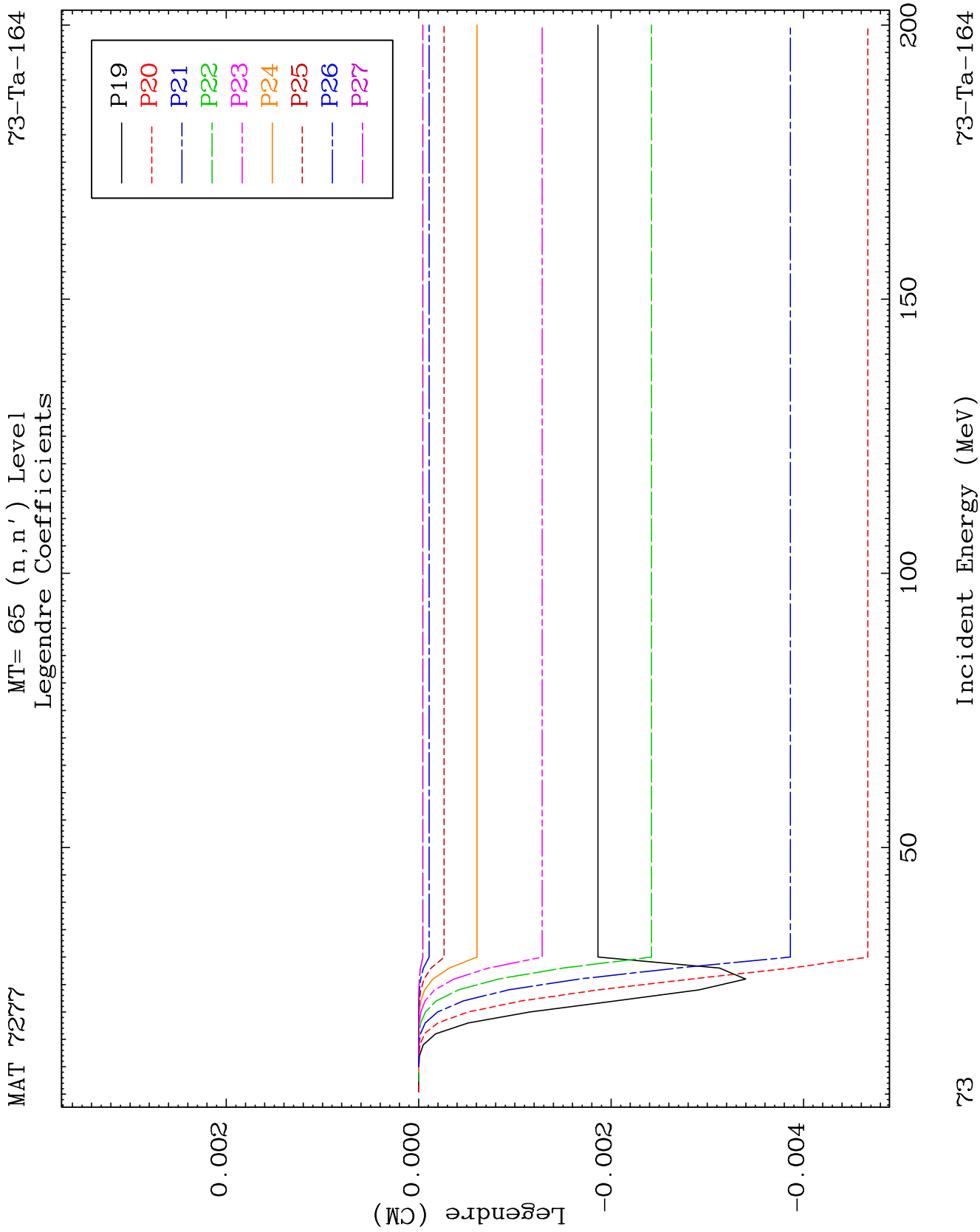








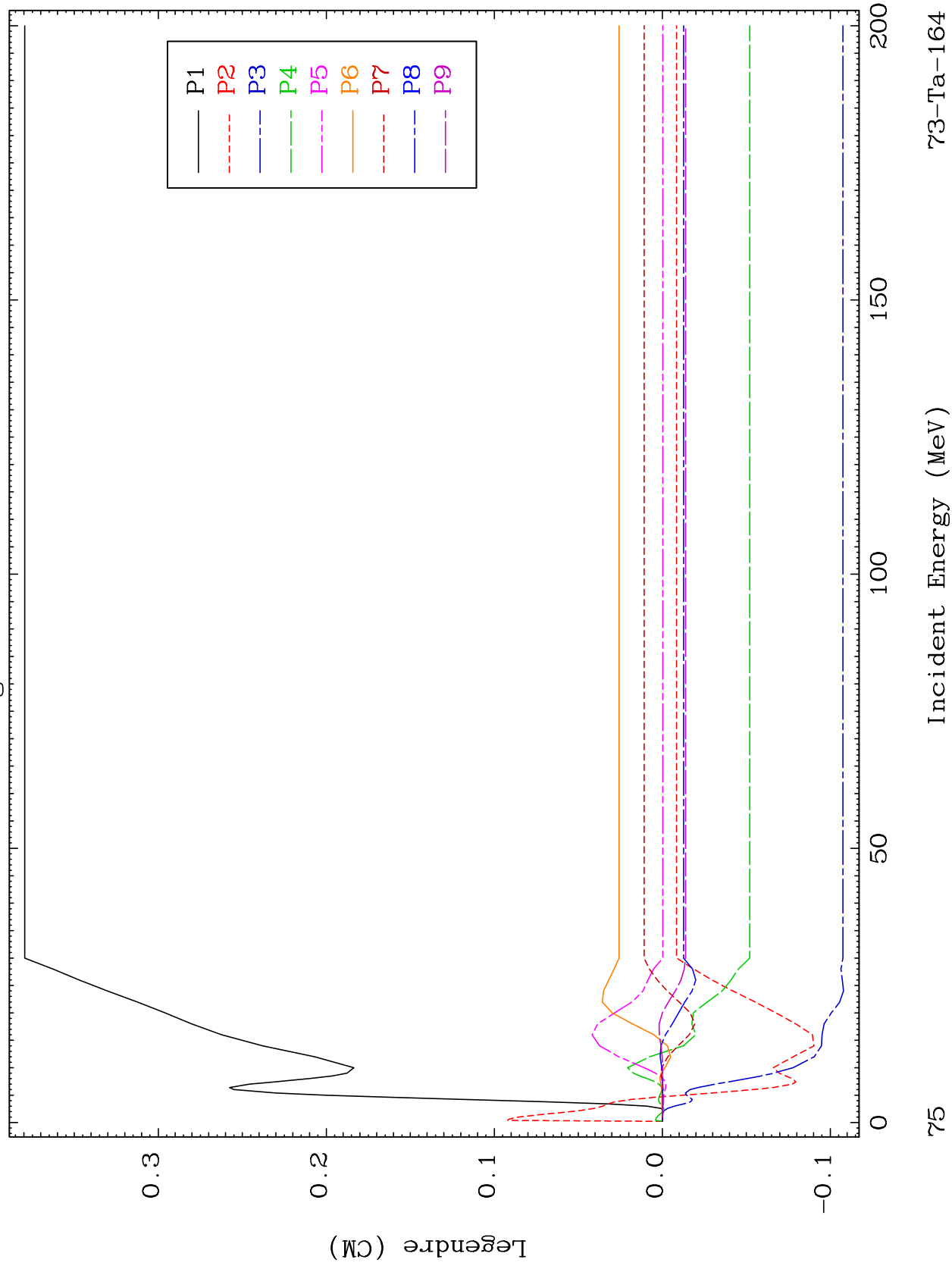


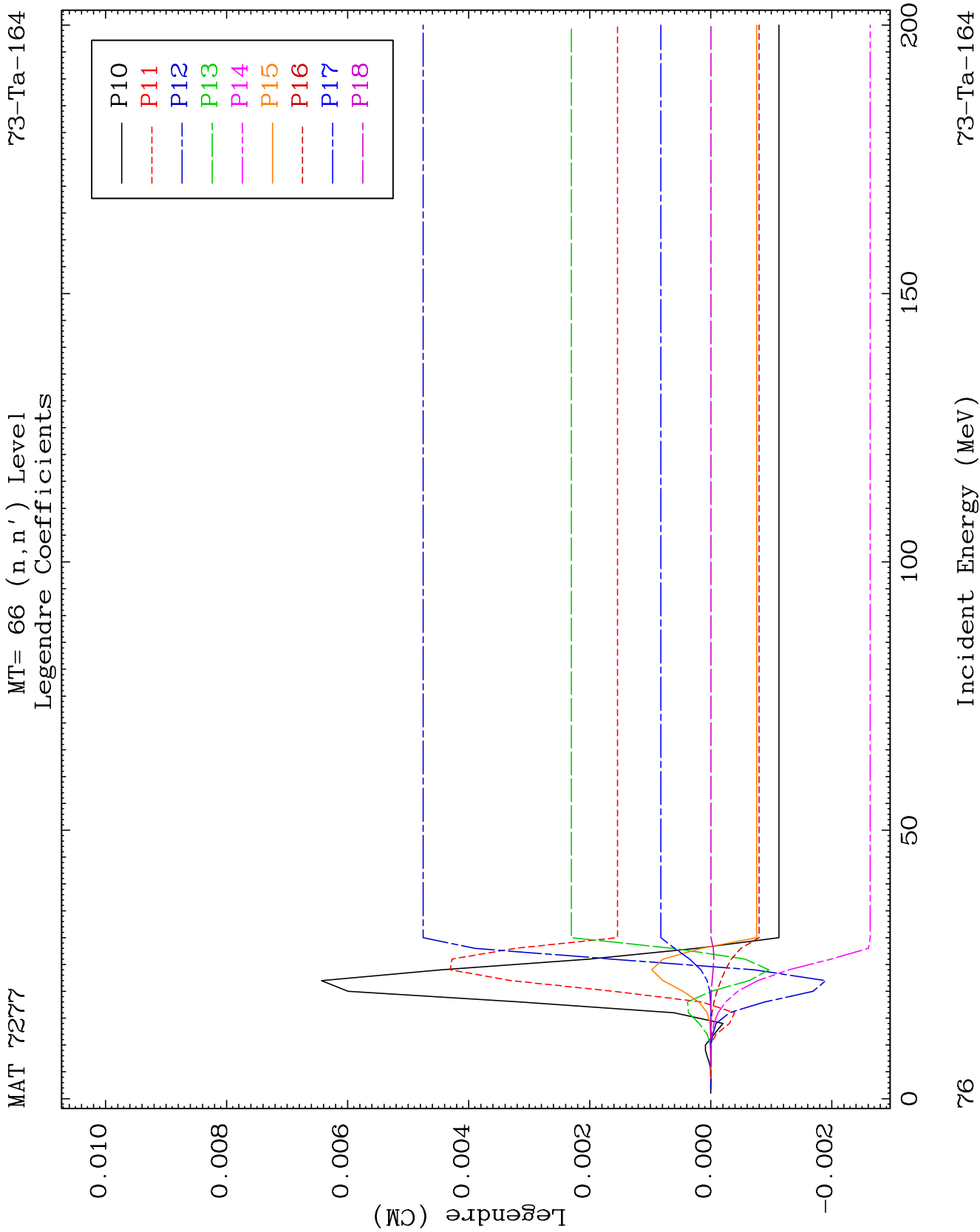


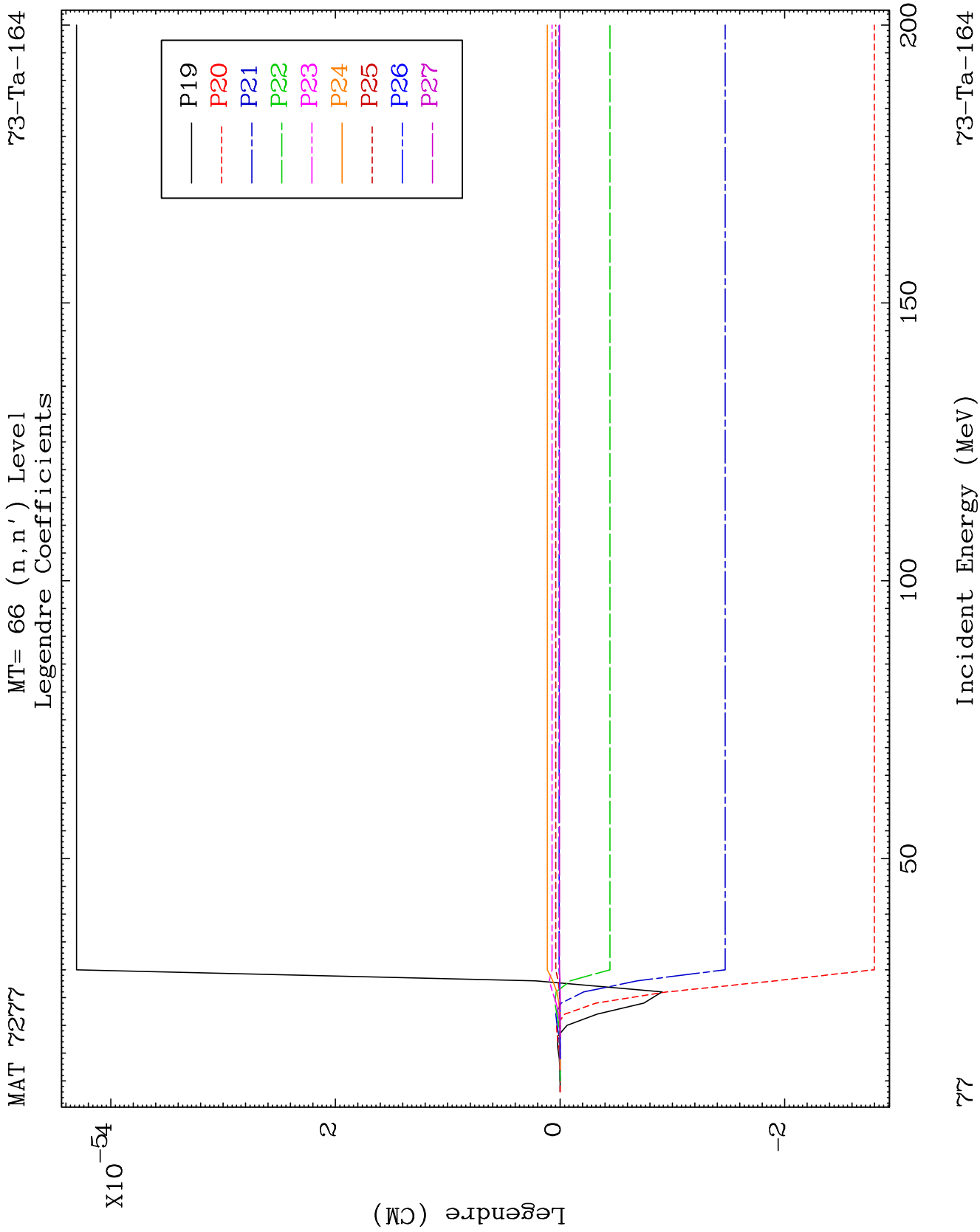
MAT 7277

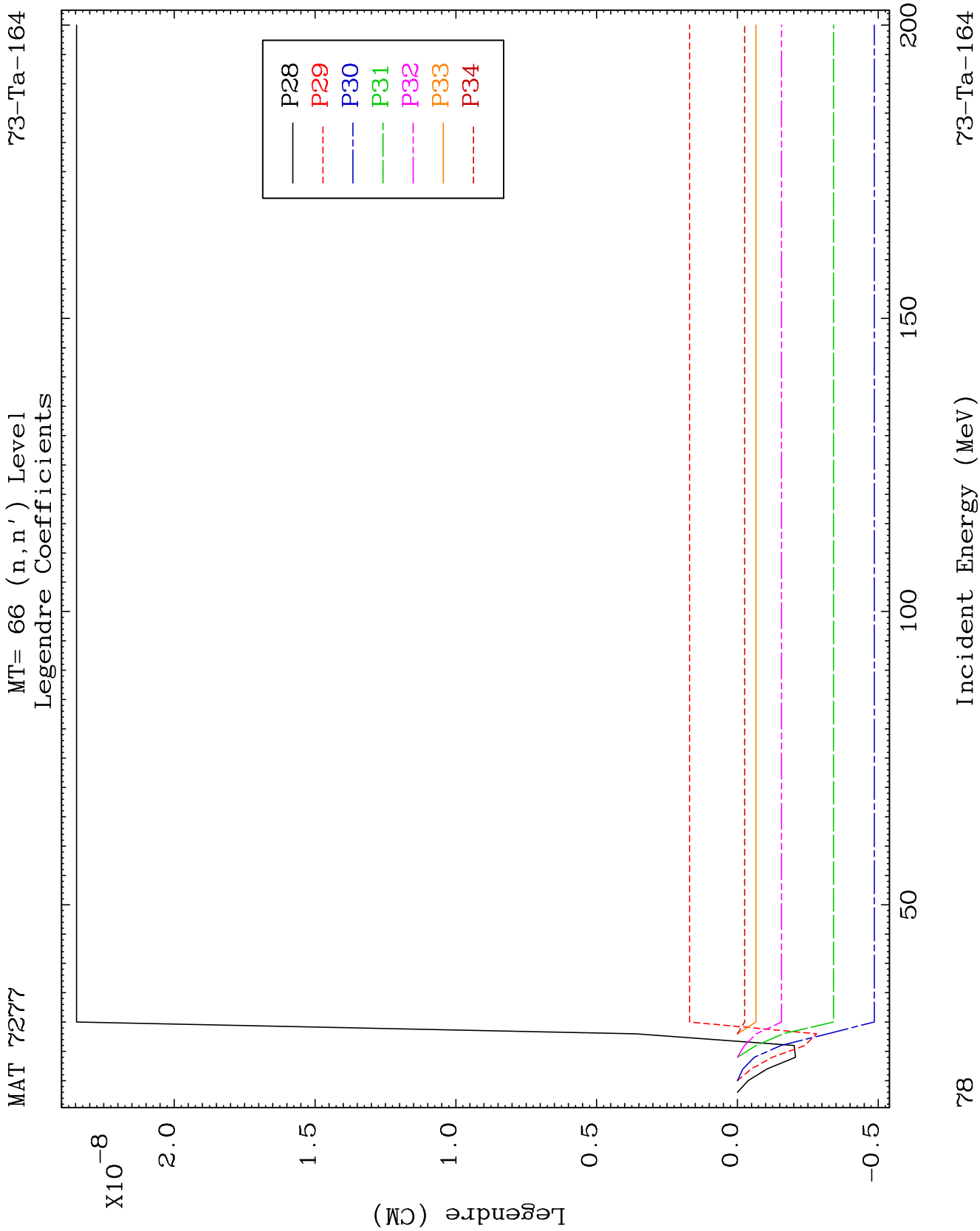
MT= 66 (n, n') Level
Legendre Coefficients

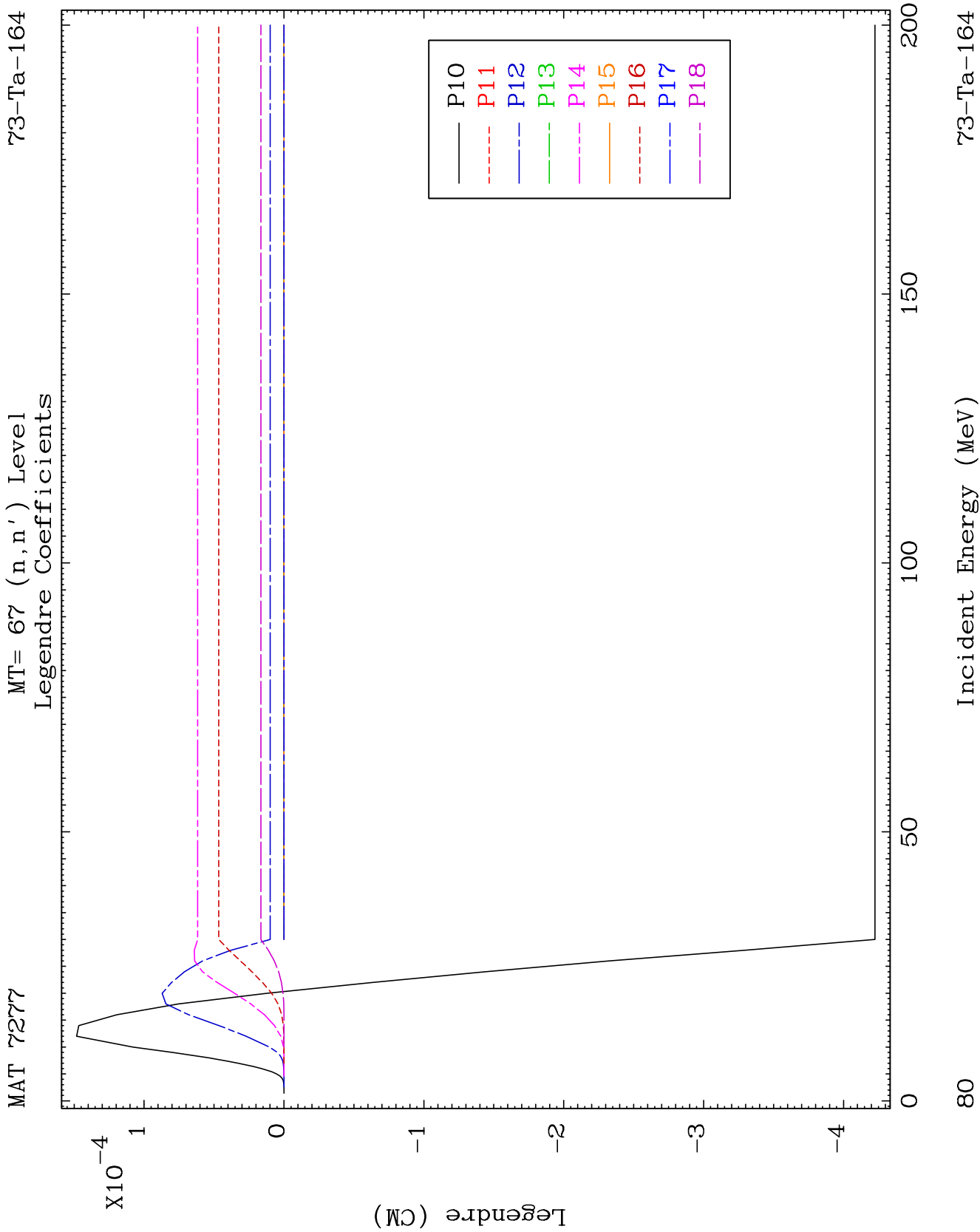
73-Ta-164

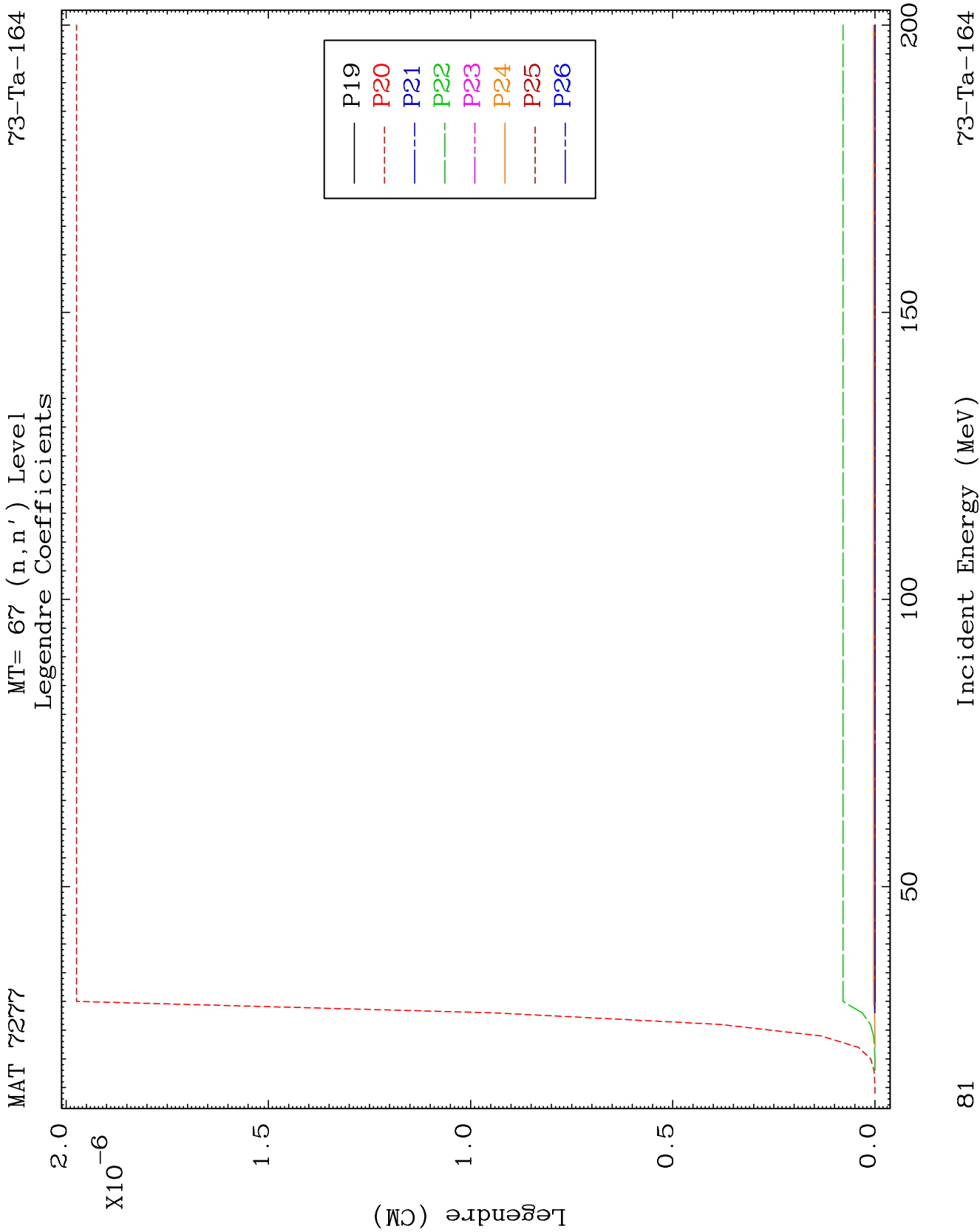








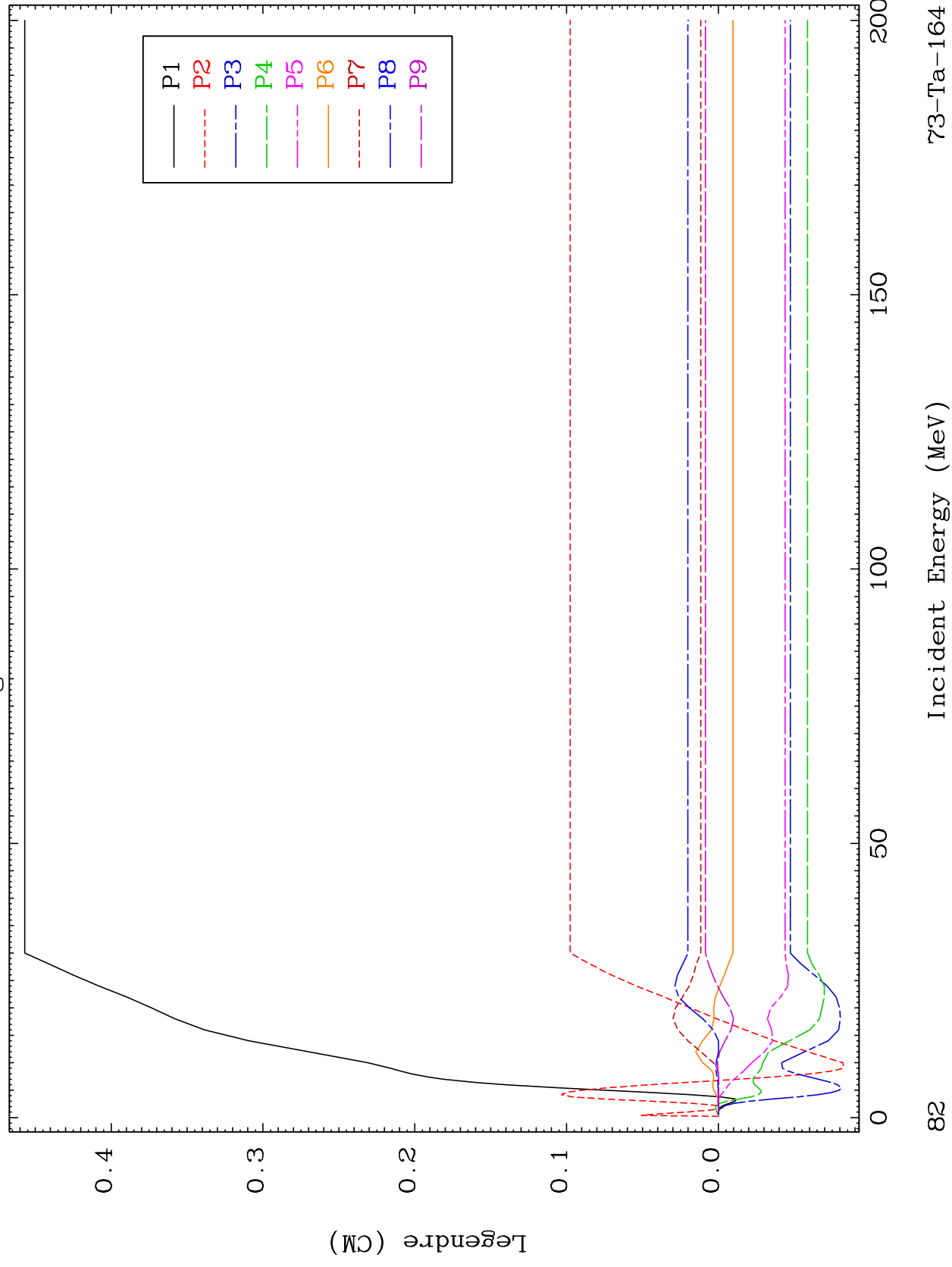


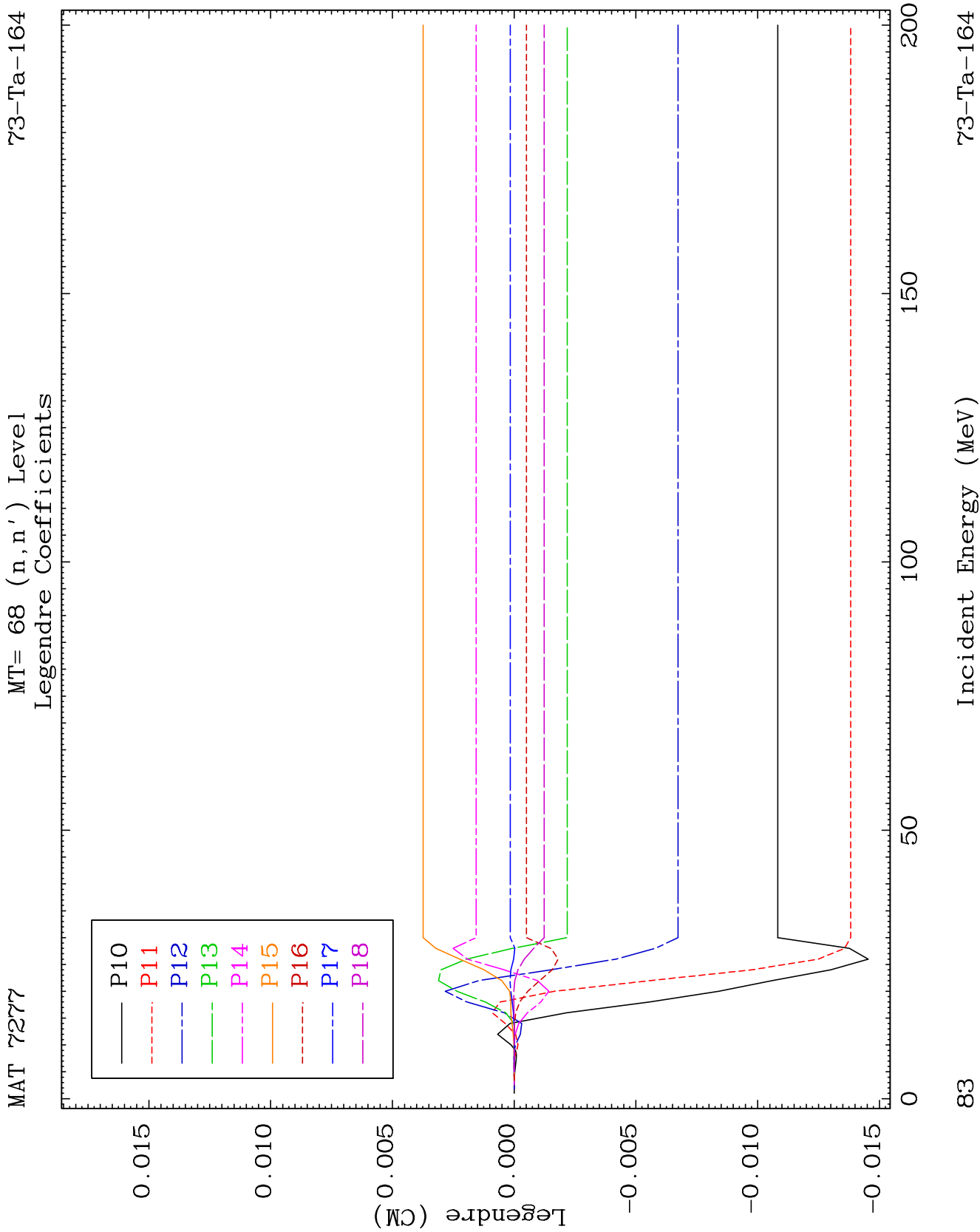


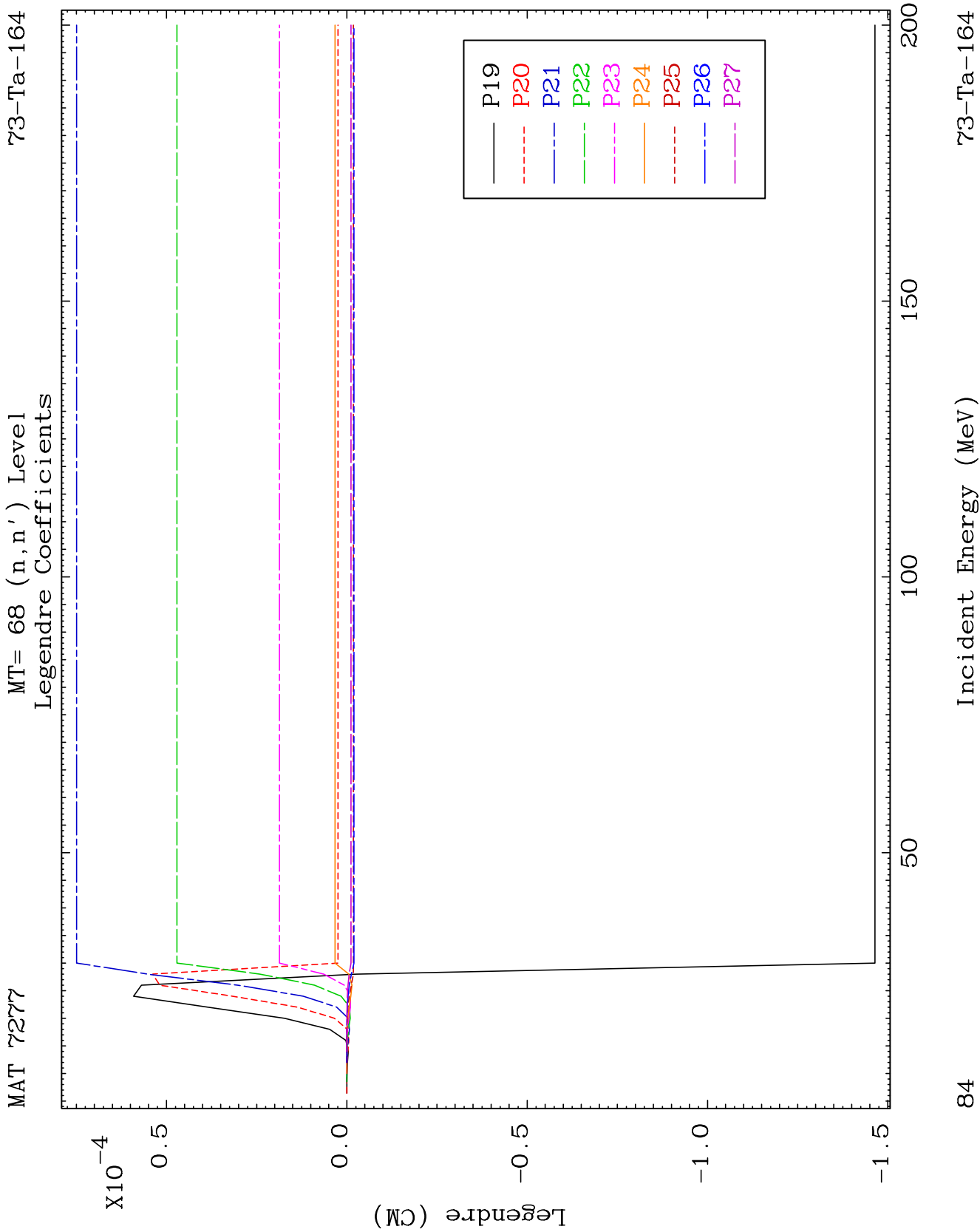
MAT 7277

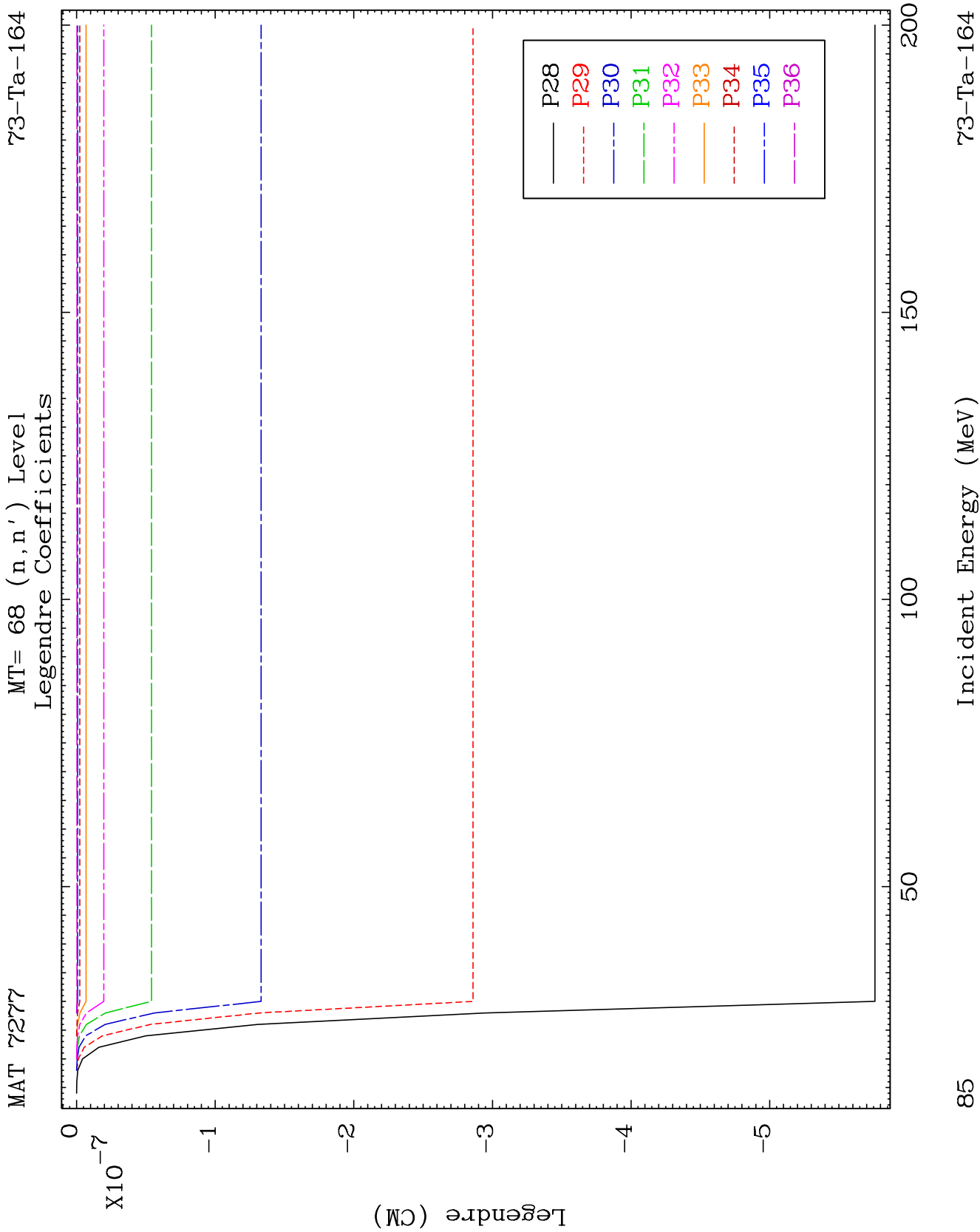
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Legendre Coefficients

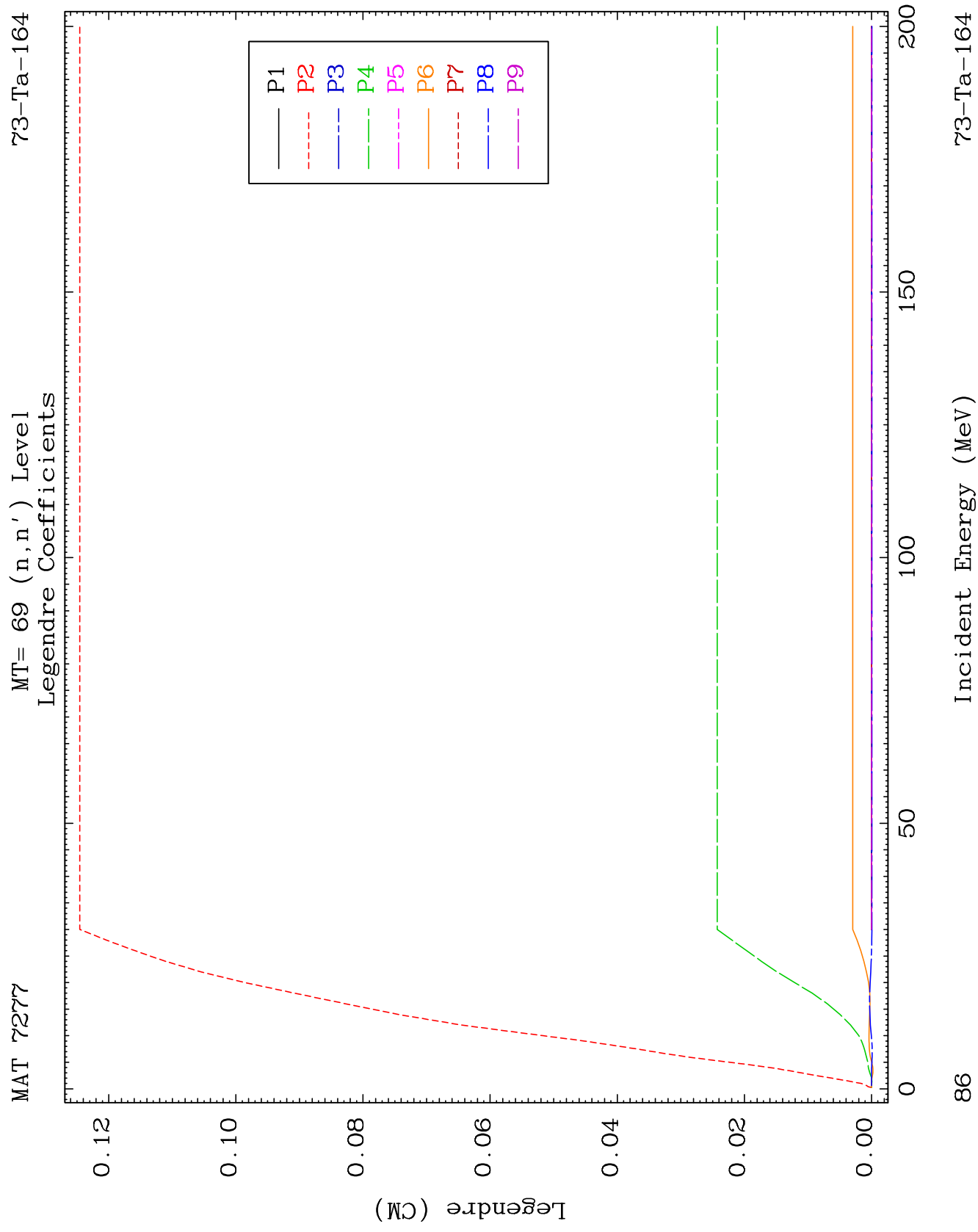
73-Ta-164

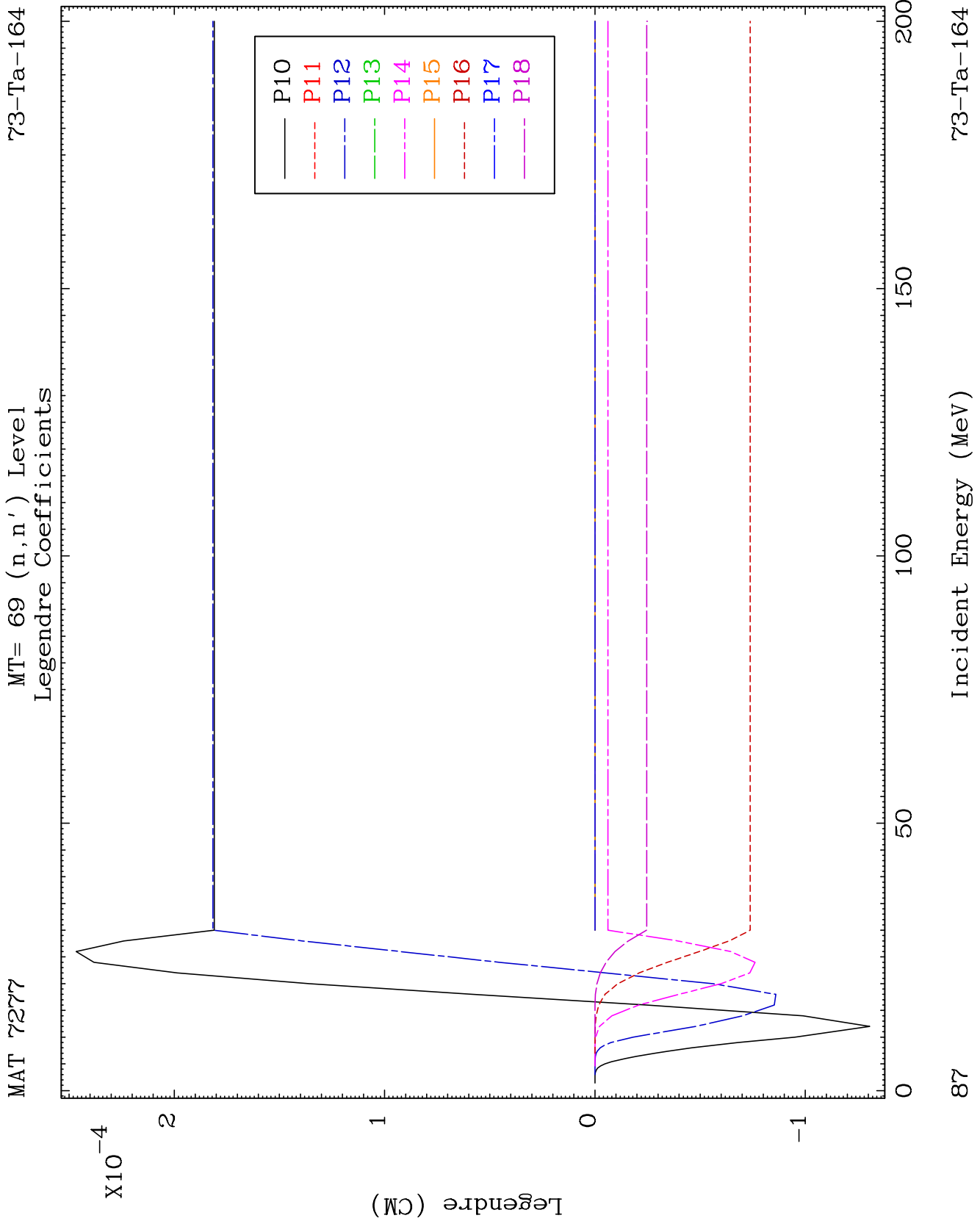


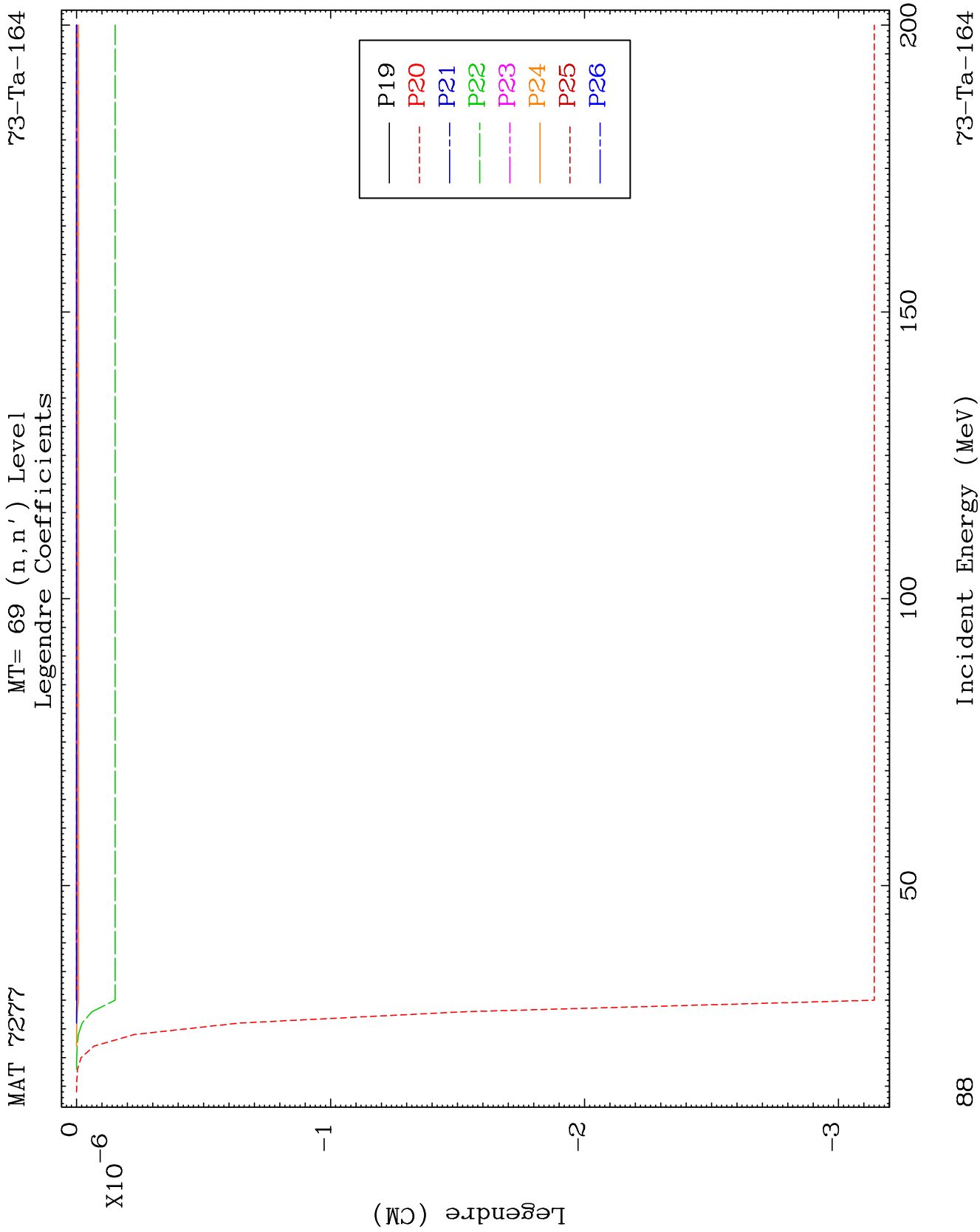


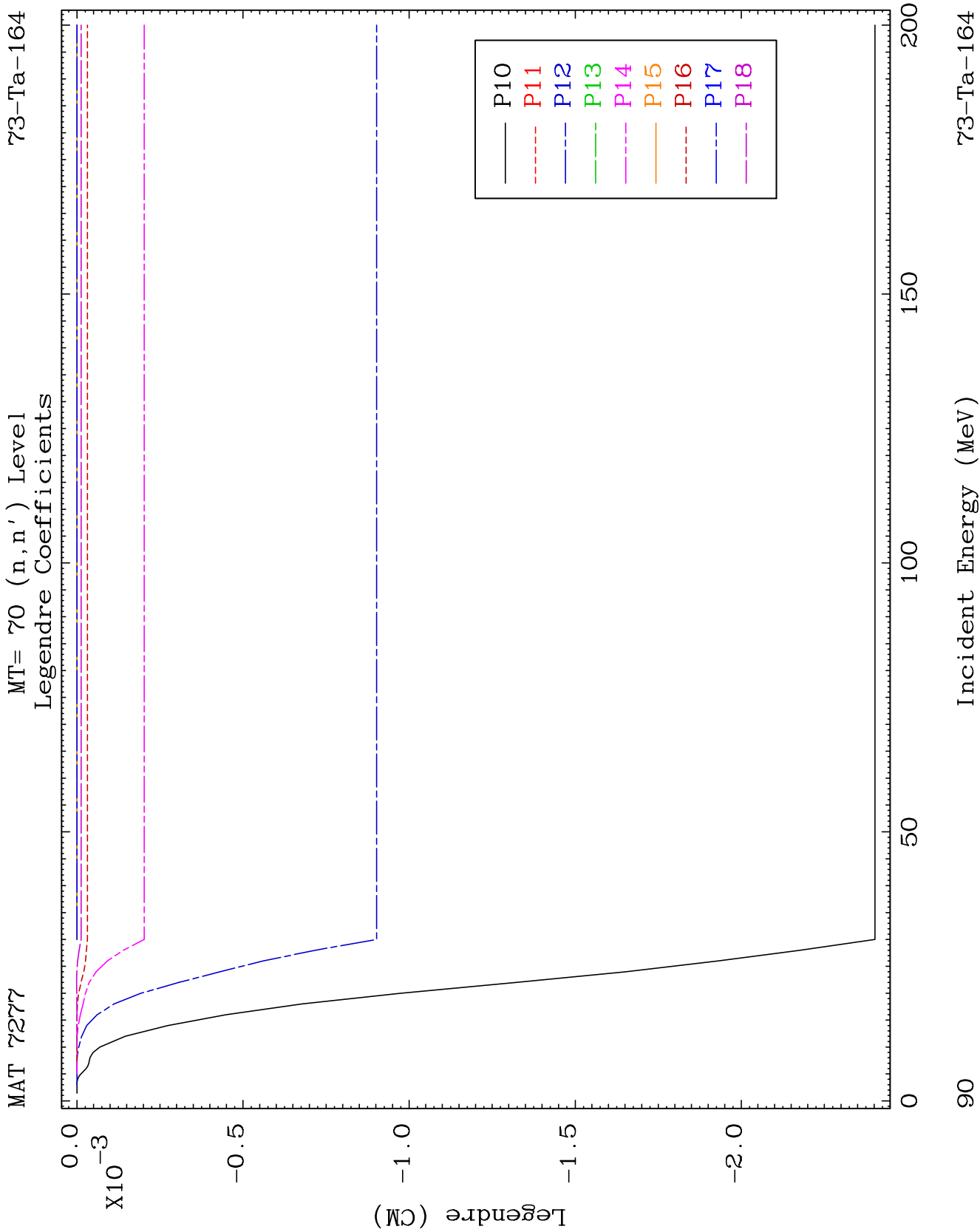


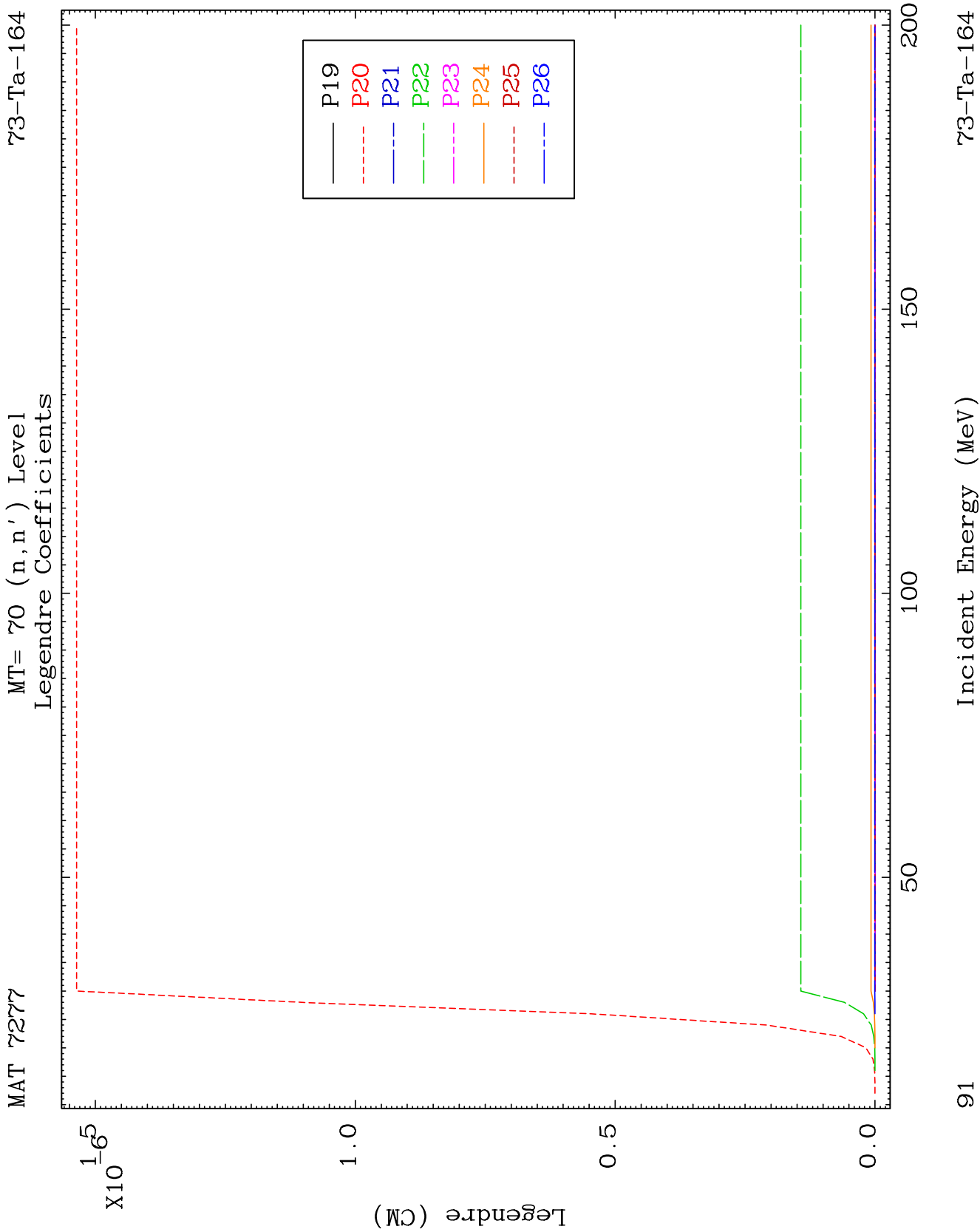


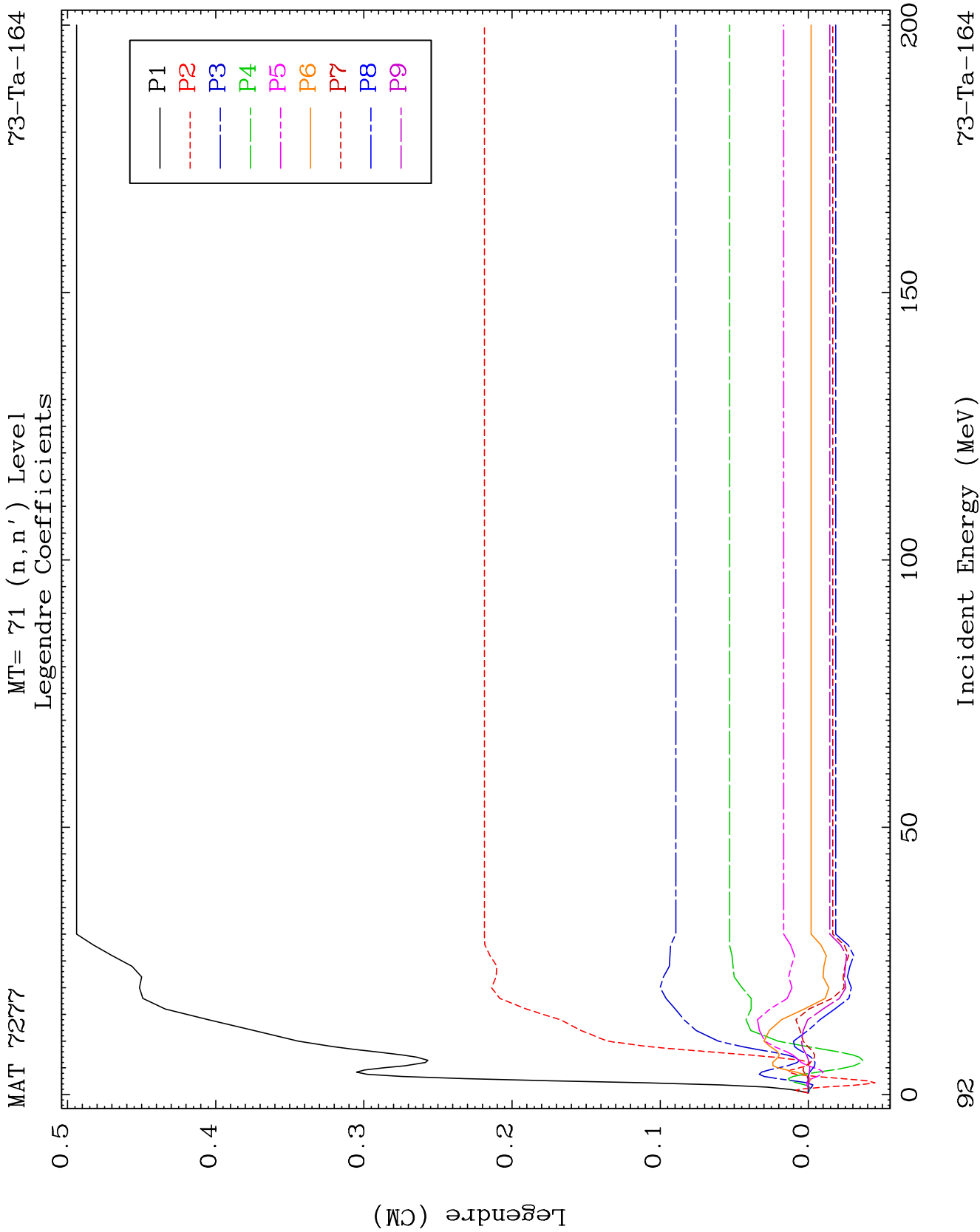


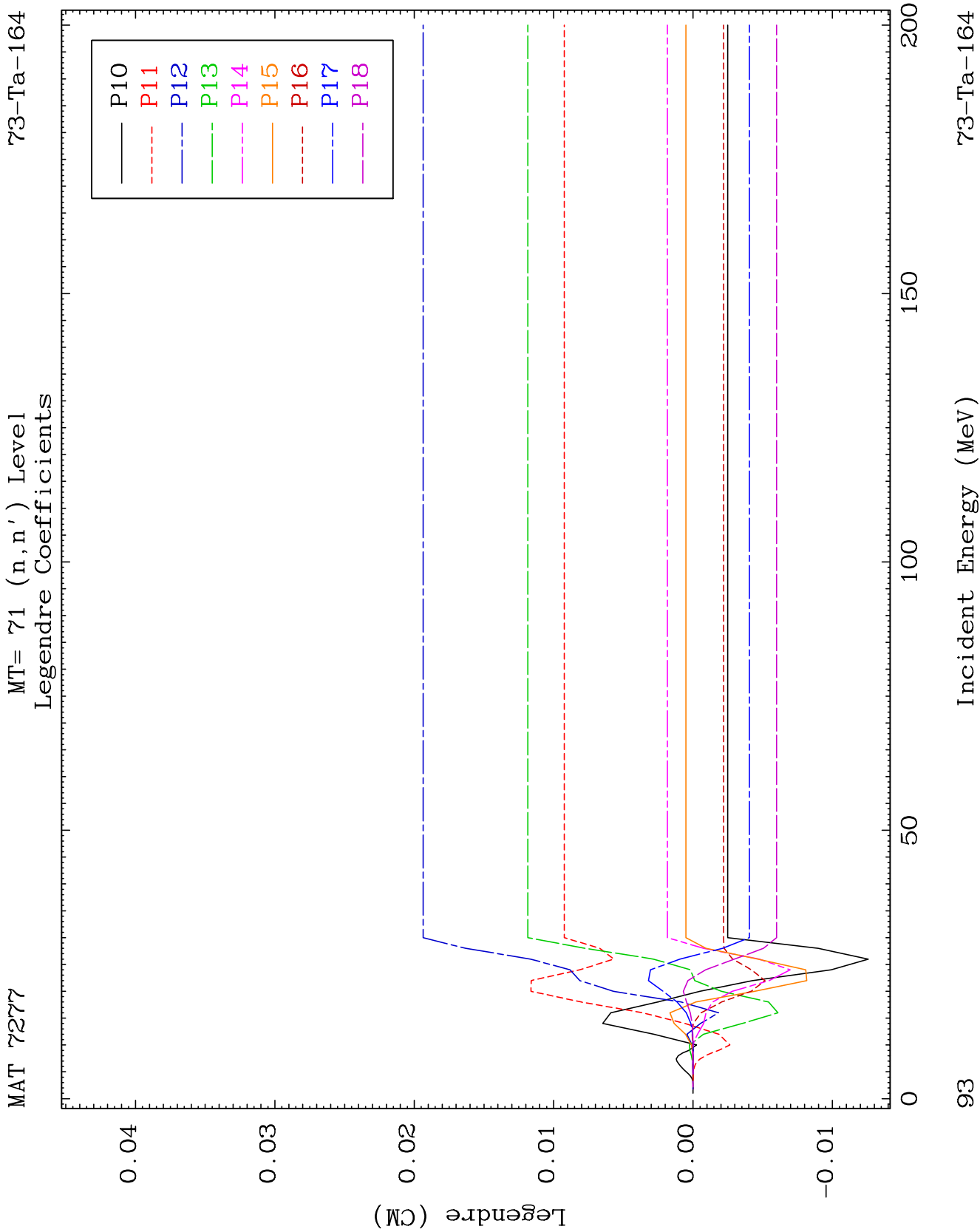








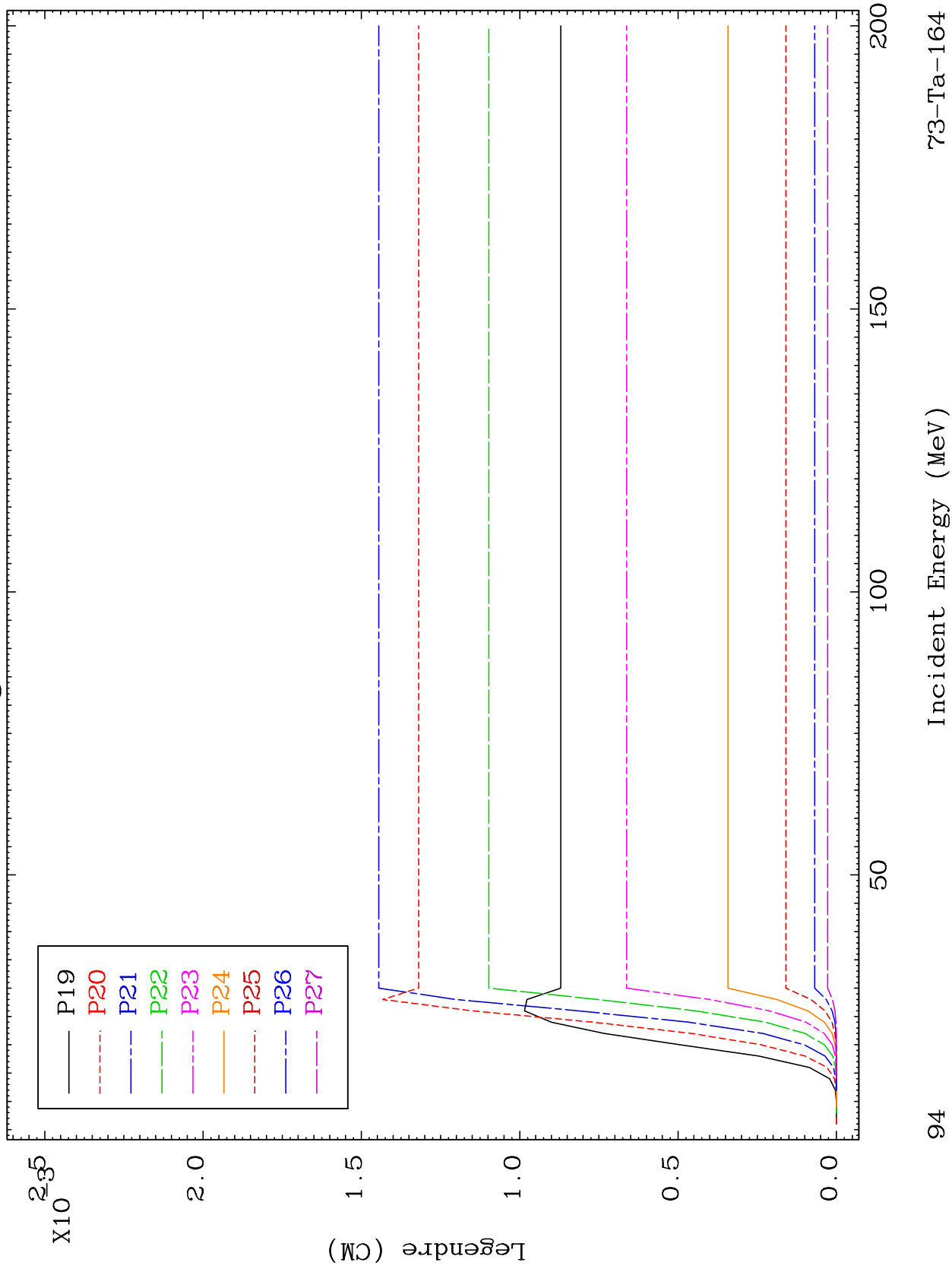


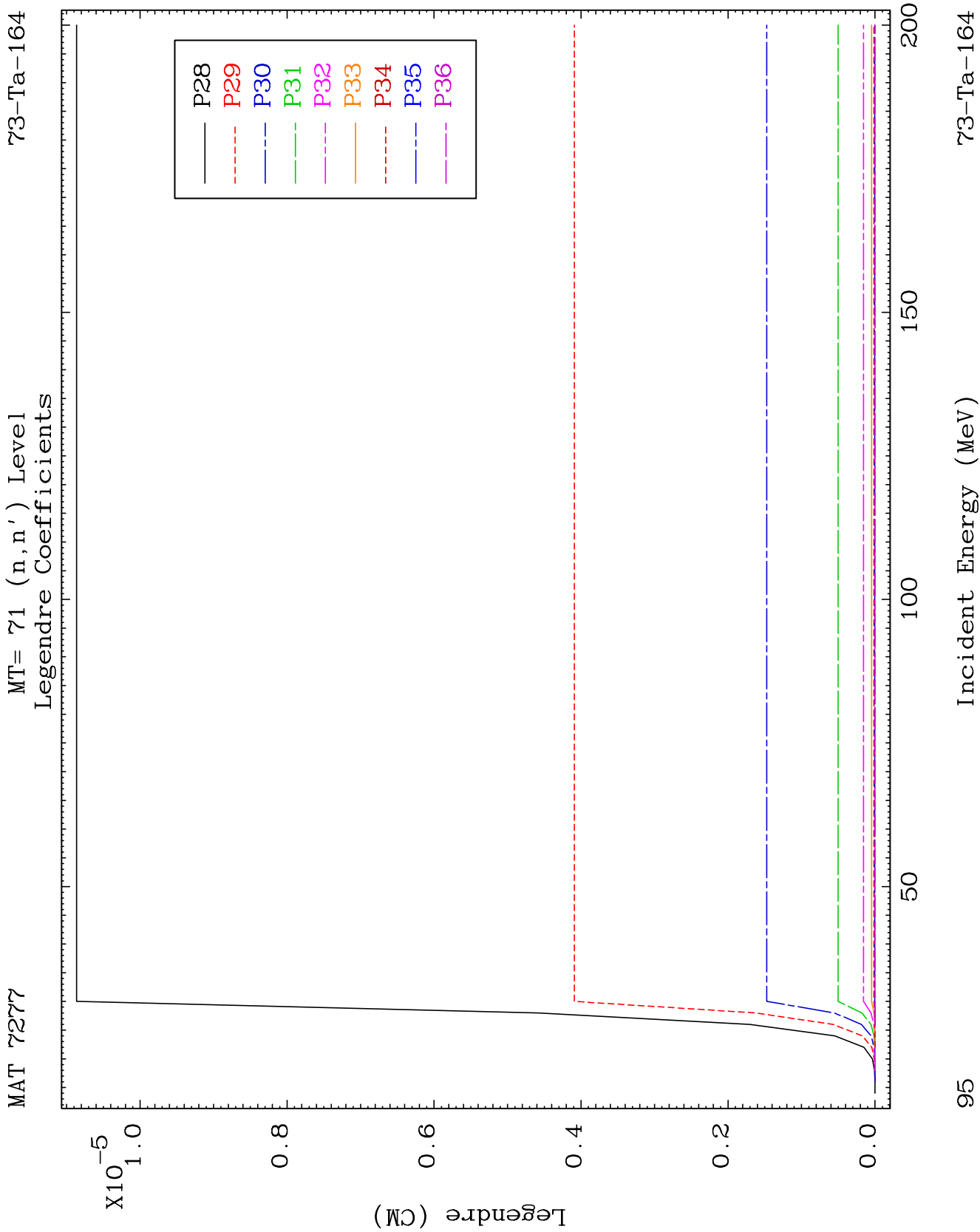


MAT 7277

MT= 71 (n, n') Level
Legendre Coefficients

73-Ta-164

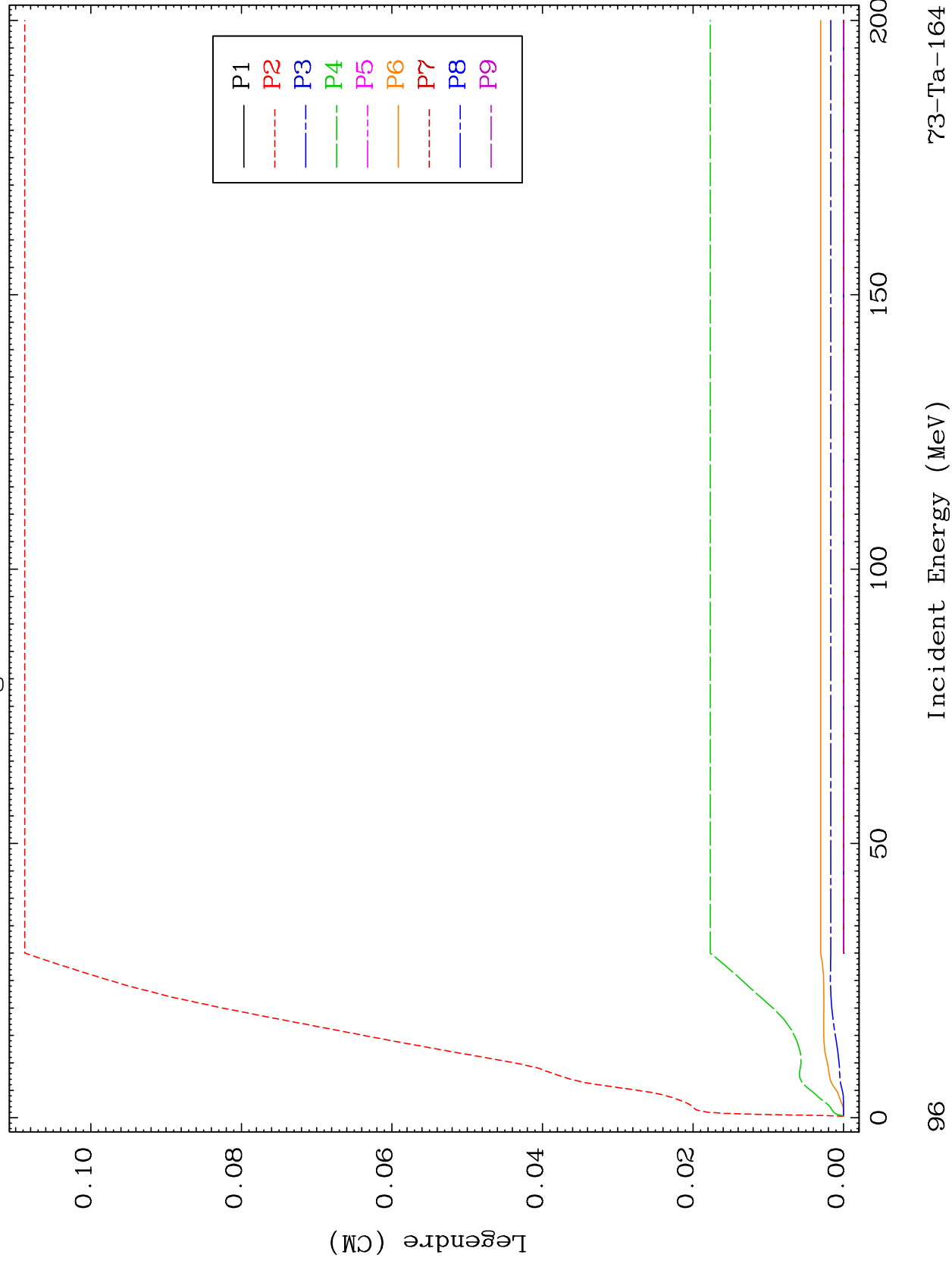


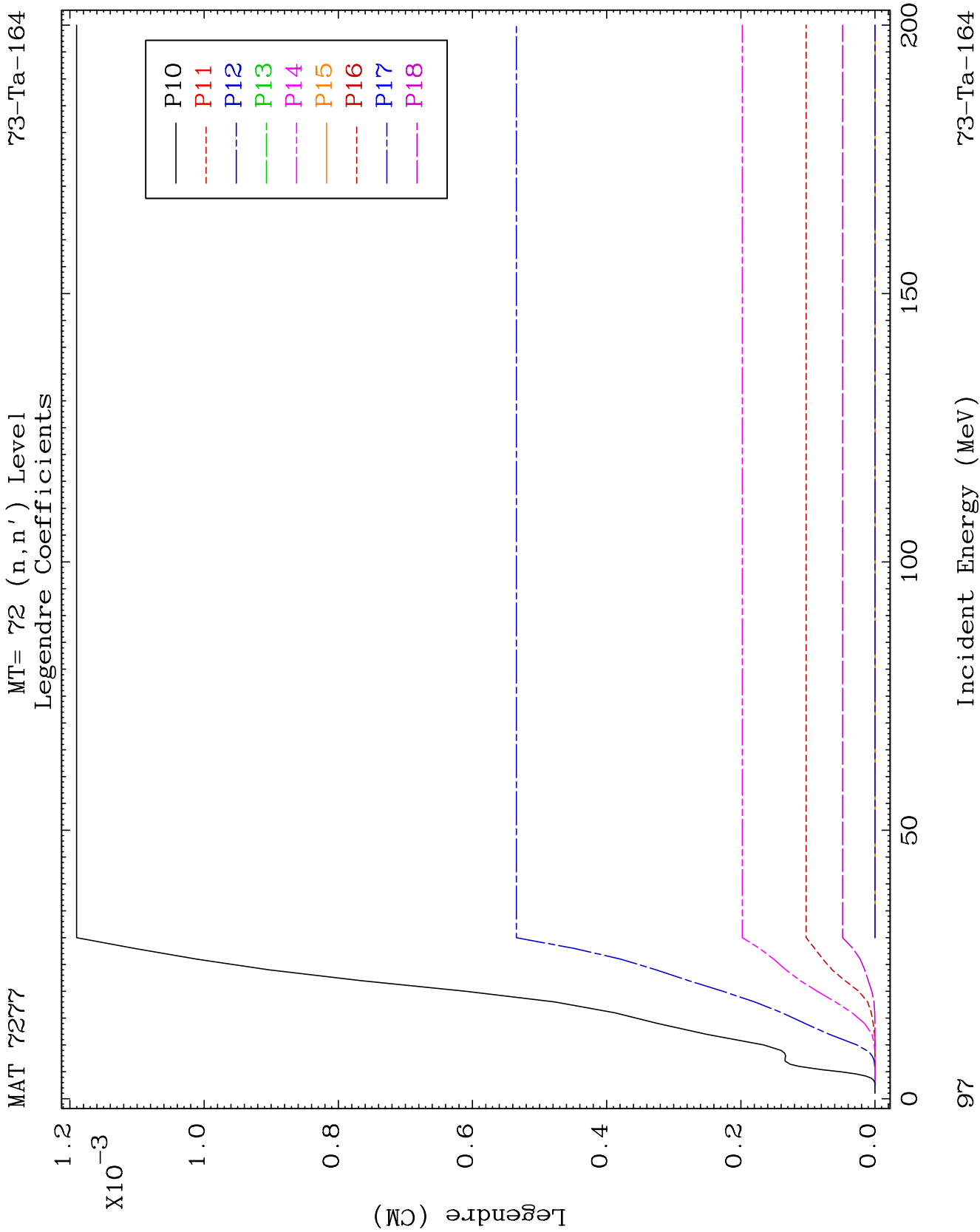


MAT 7277

MT= 72 (n, n') Level
Legendre Coefficients

73-Ta-164

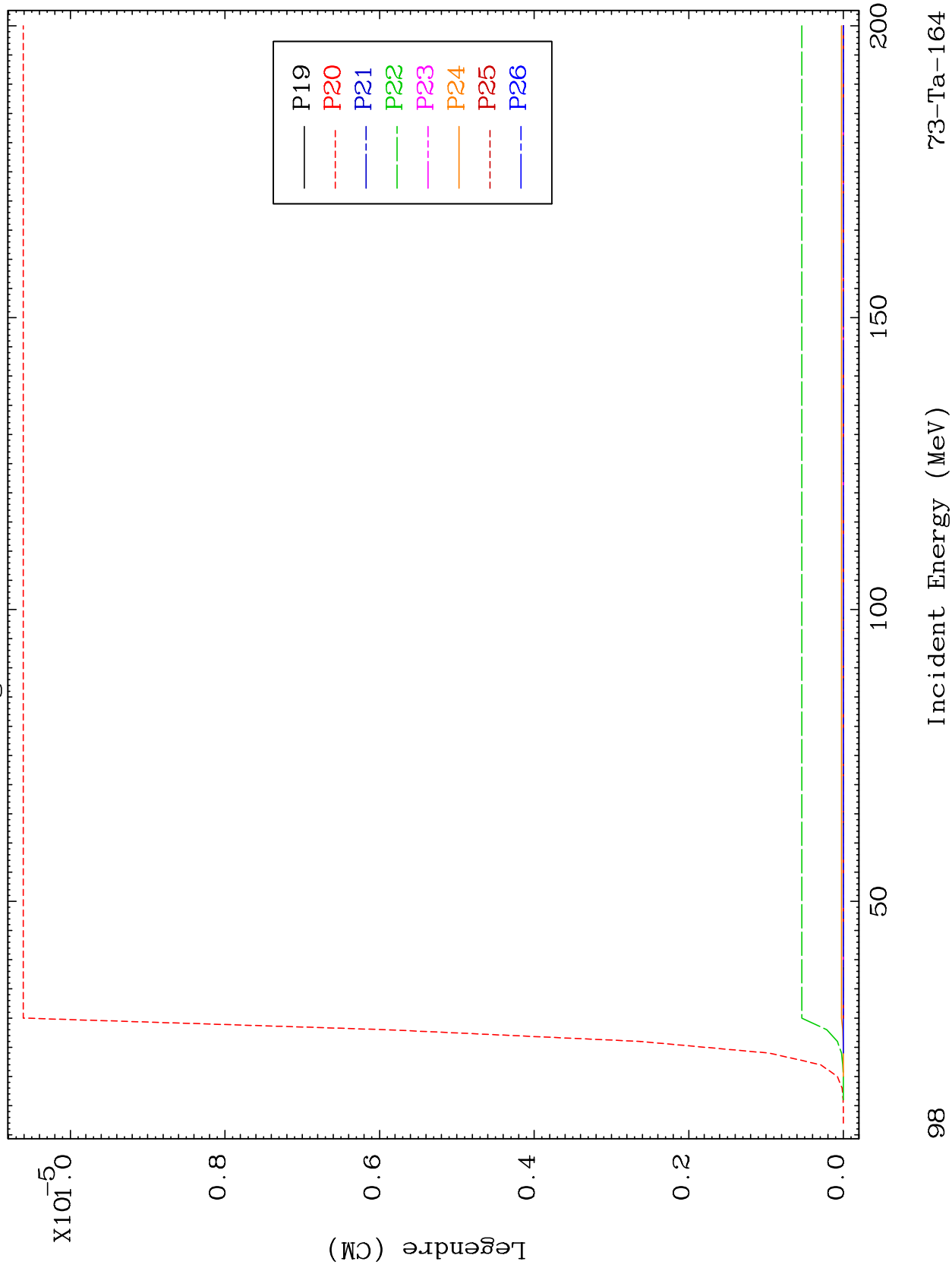




MAT 7277

MT= 72 (n, n') Level
Legendre Coefficients

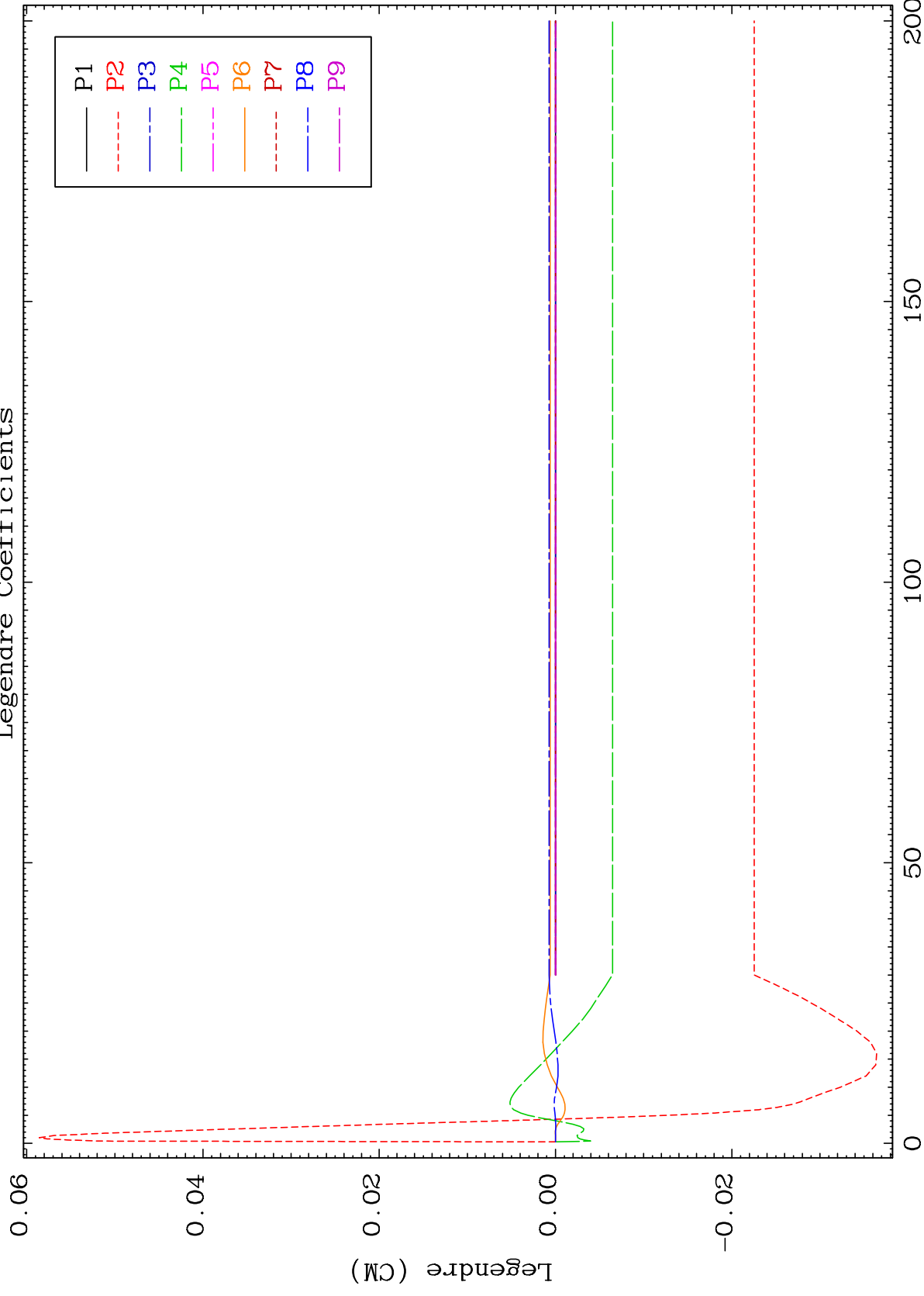
73-Ta-164



MAT 7277

MT= 73 (n, n') Level
Legendre Coefficients

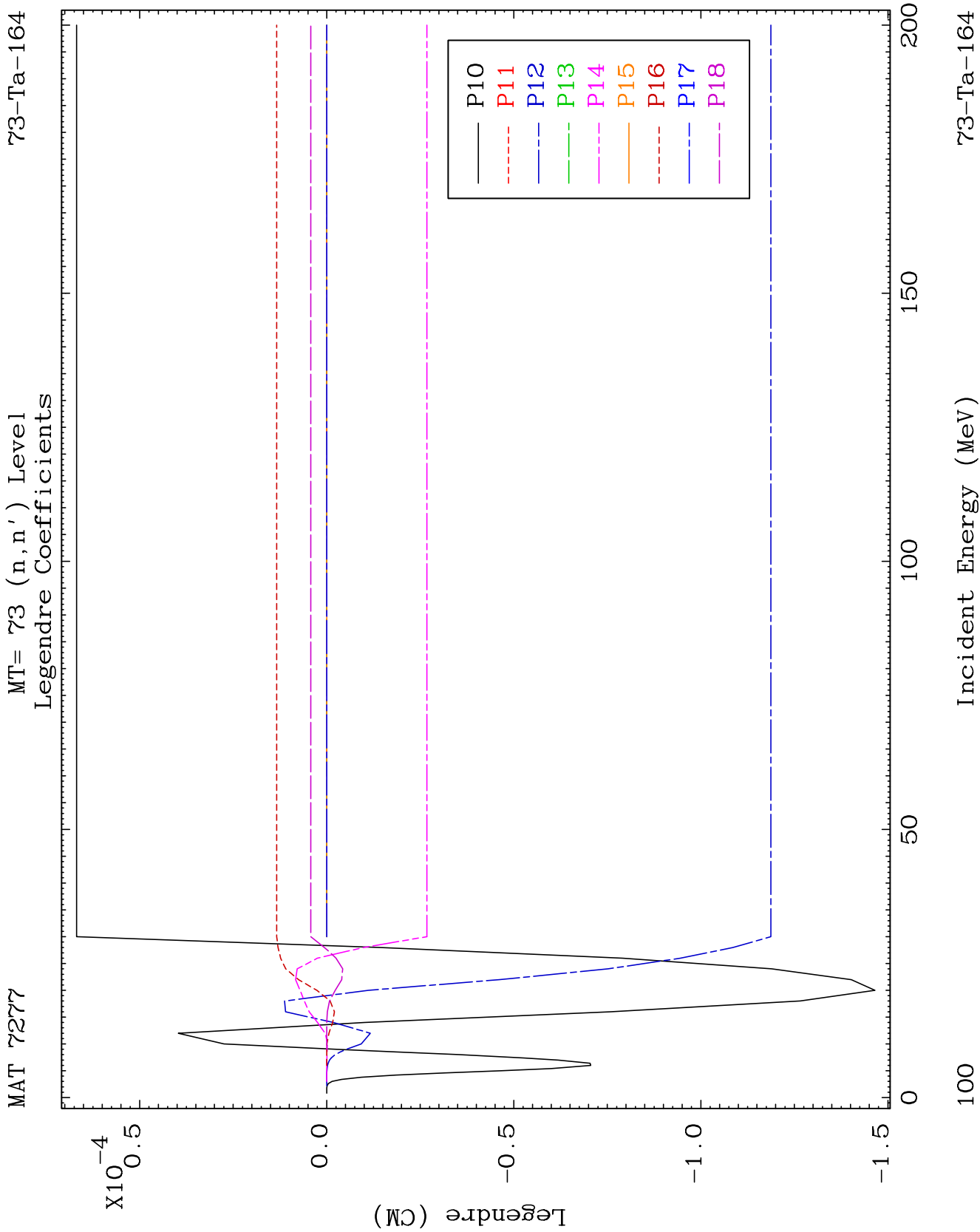
73-Ta-164

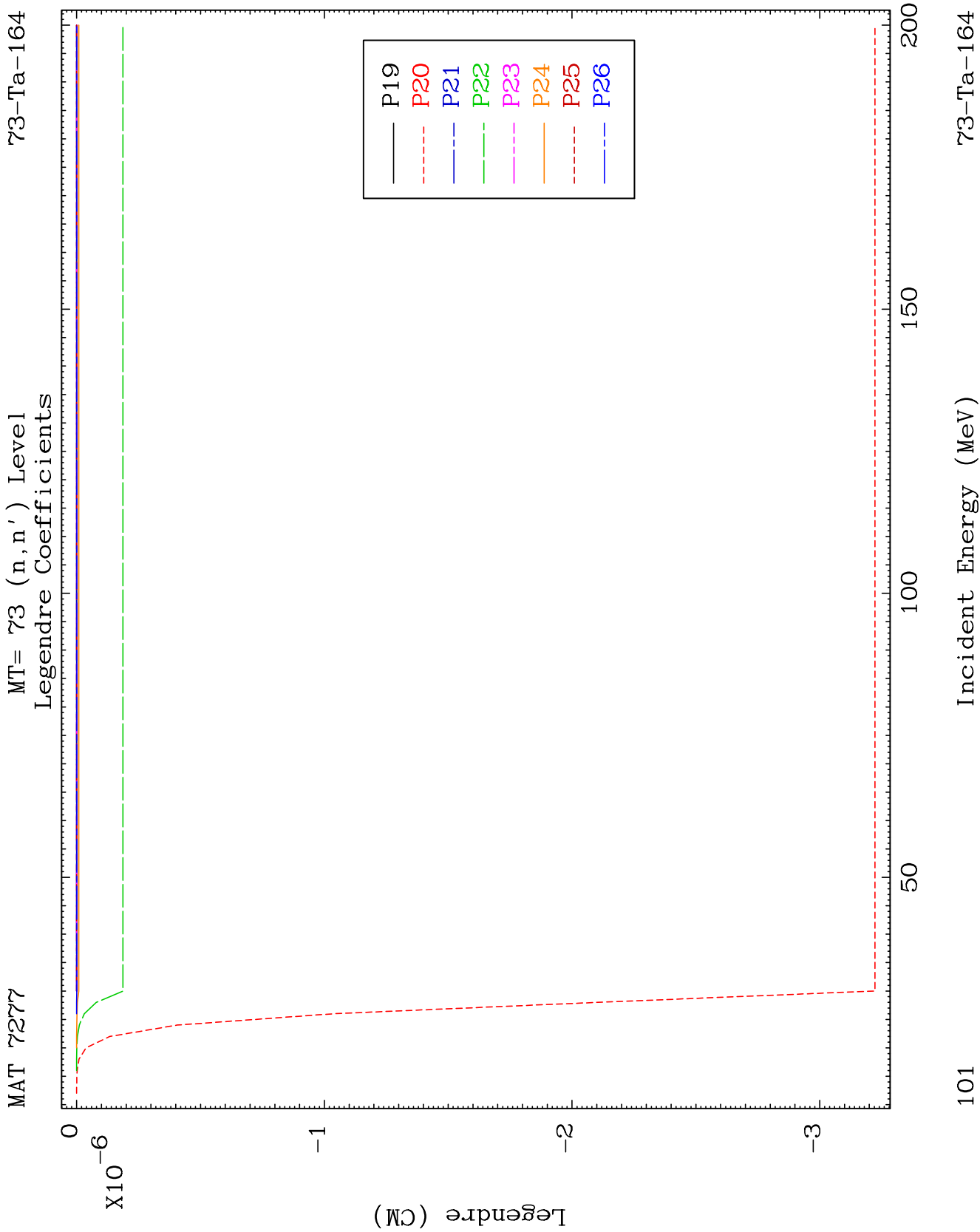


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Incident Energy (MeV)

73-Ta-164

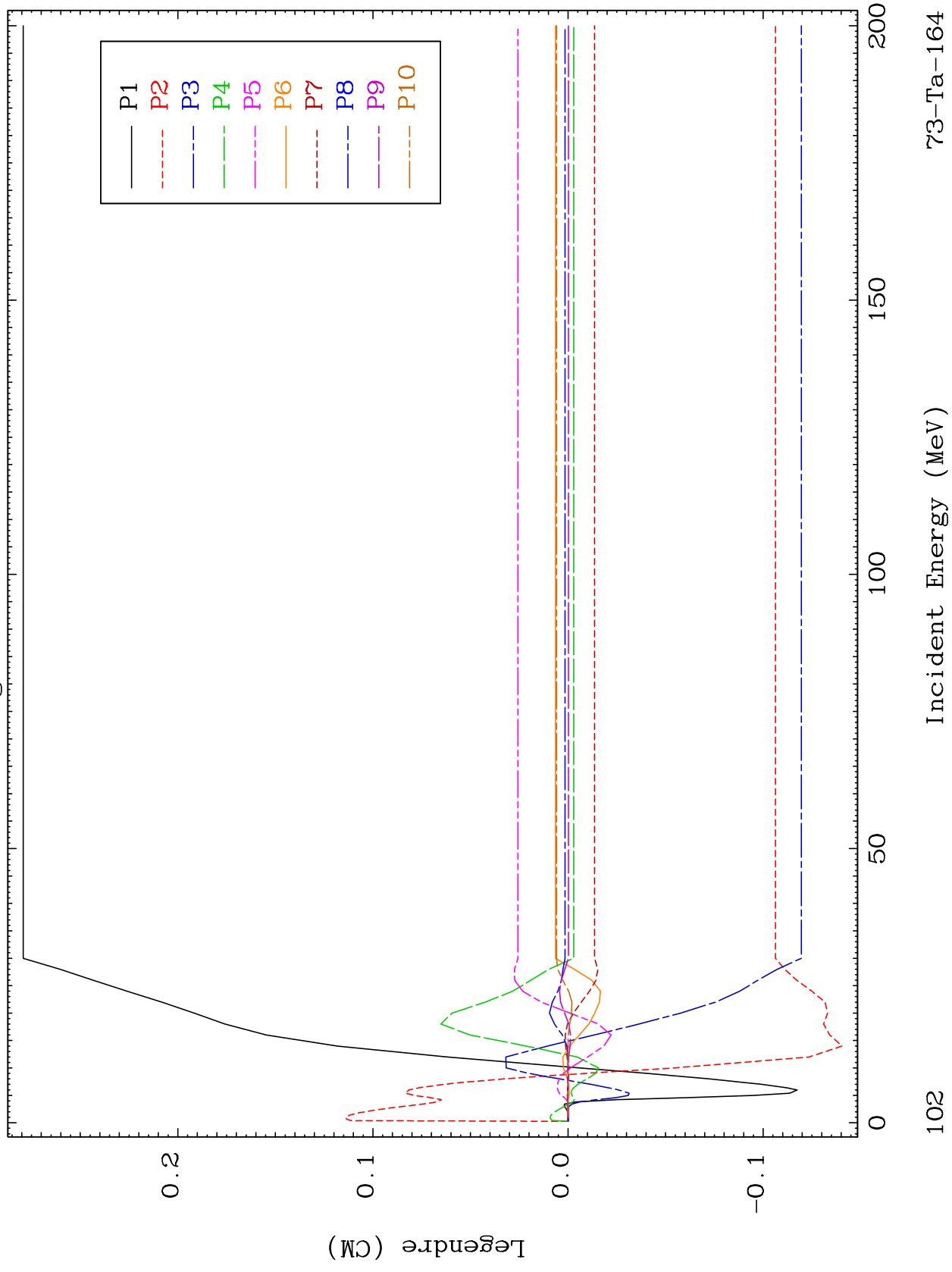


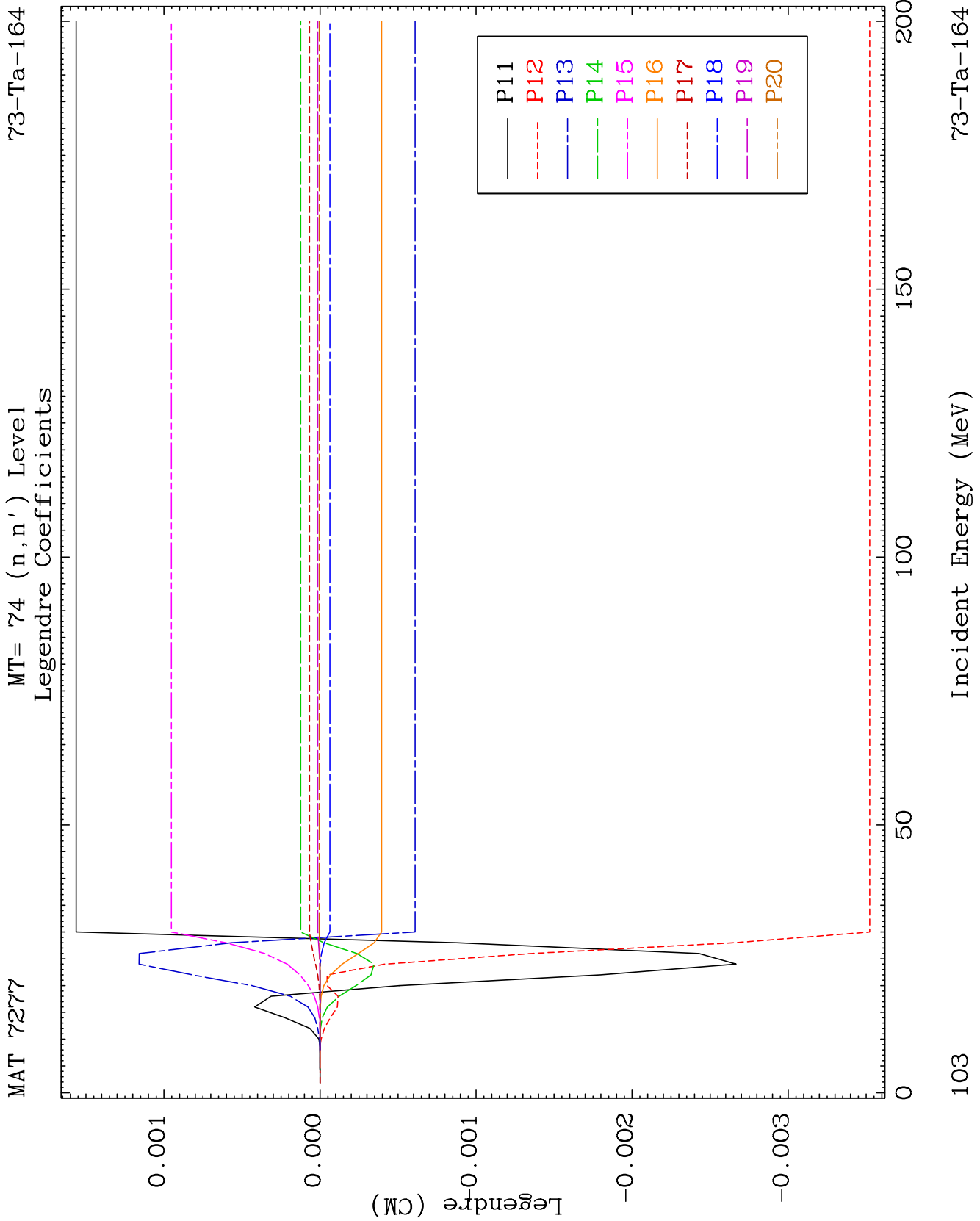


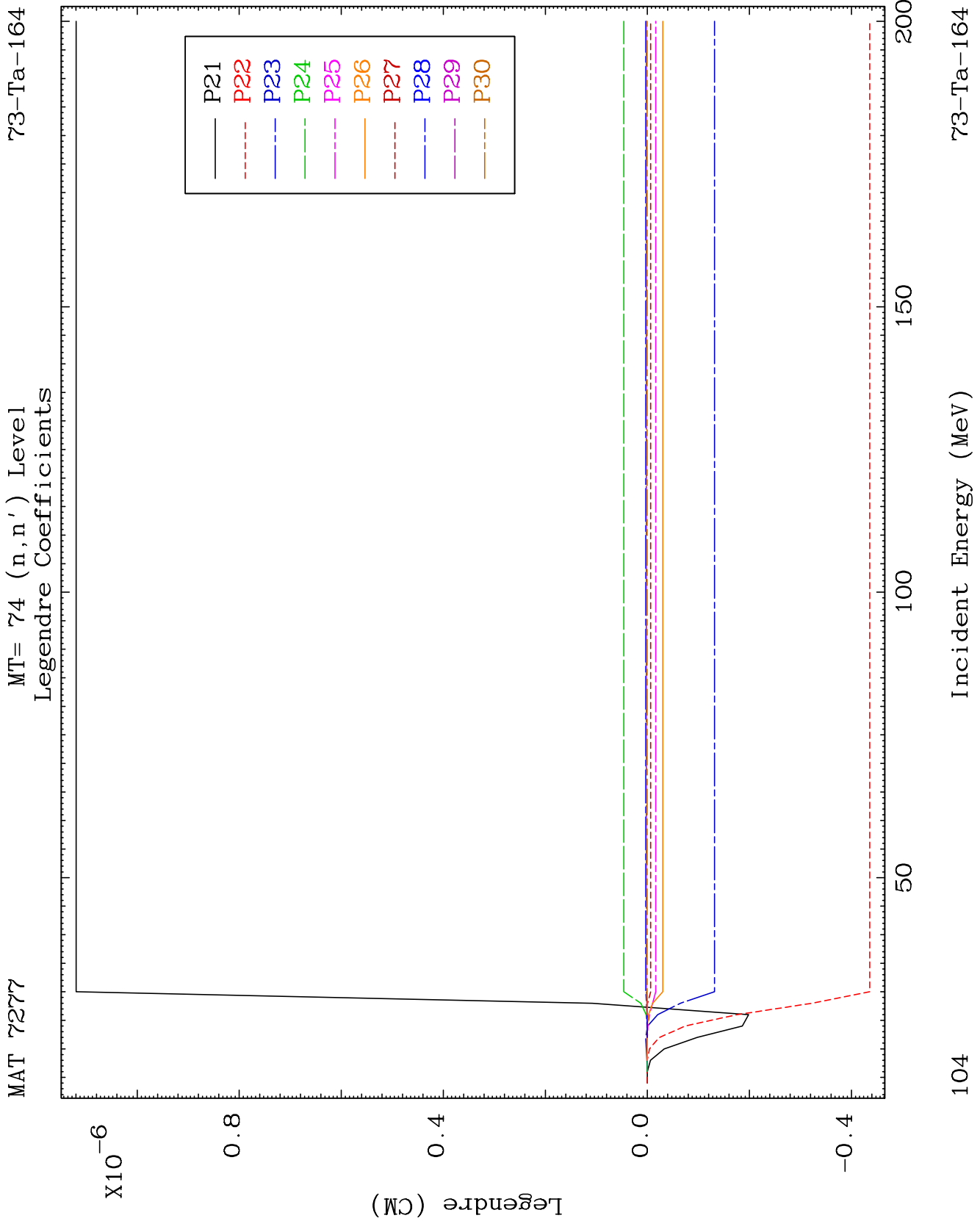
MAT 7277

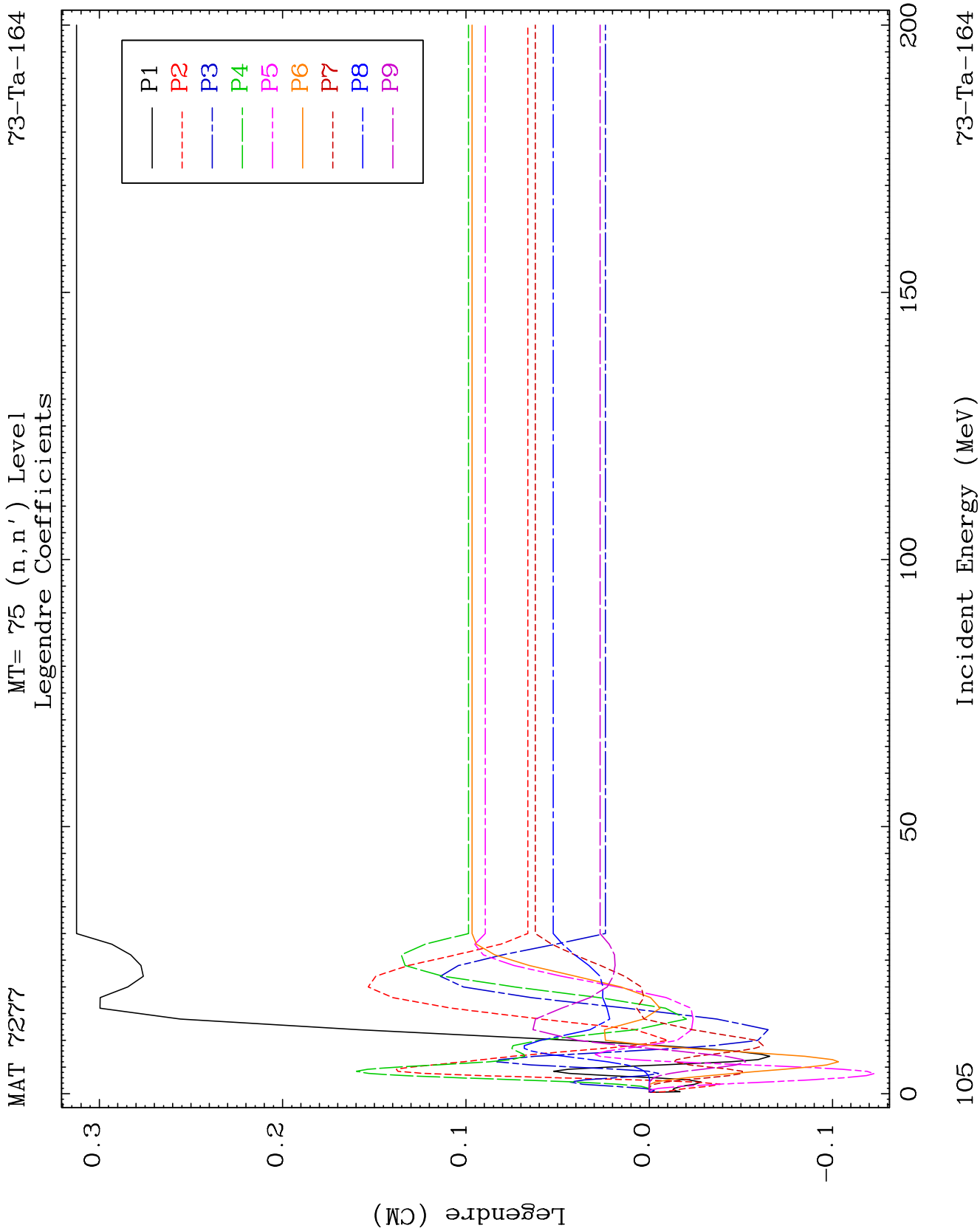
MT= 74 (n, n') Level
Legendre Coefficients

73-Ta-164





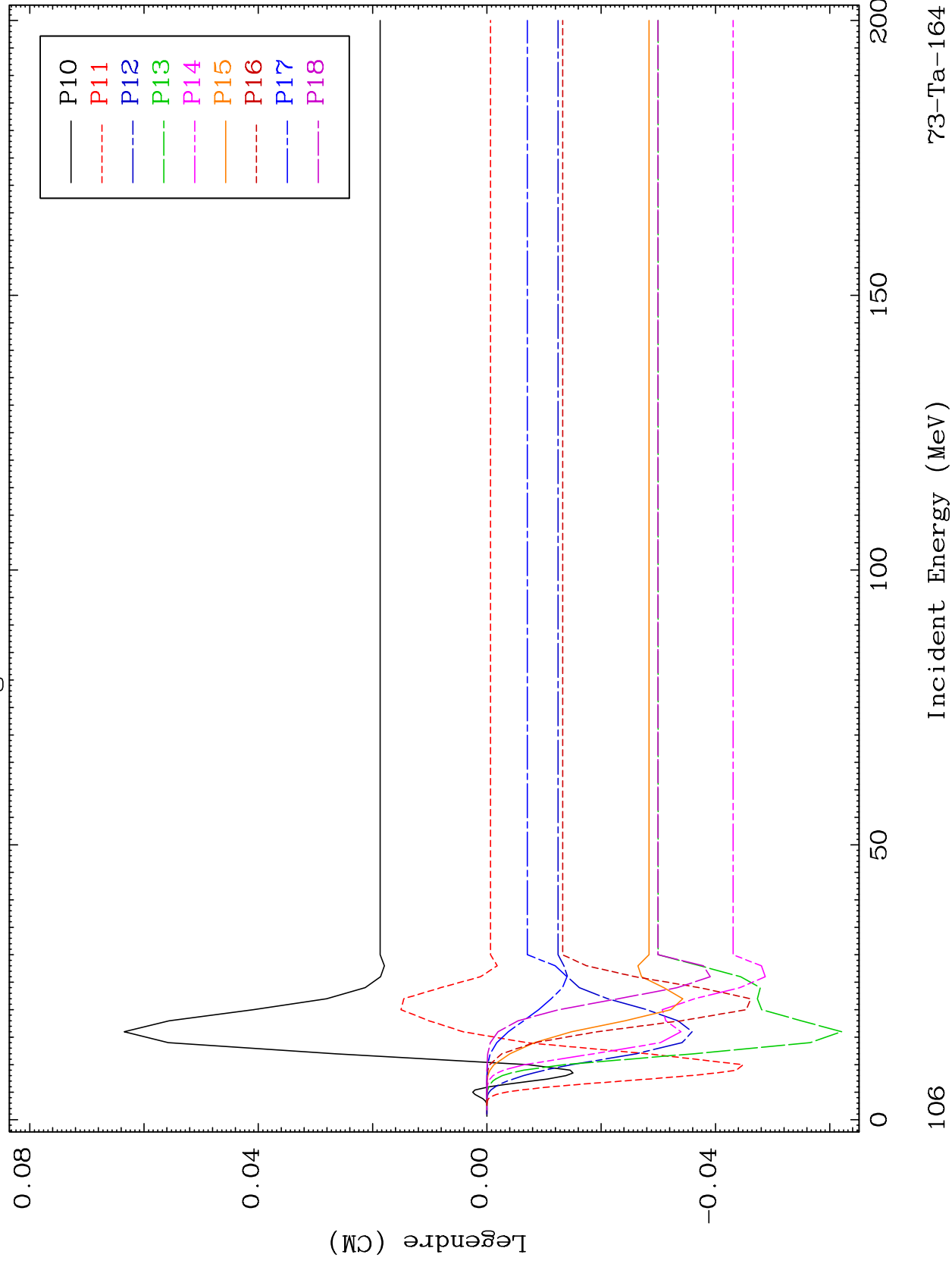


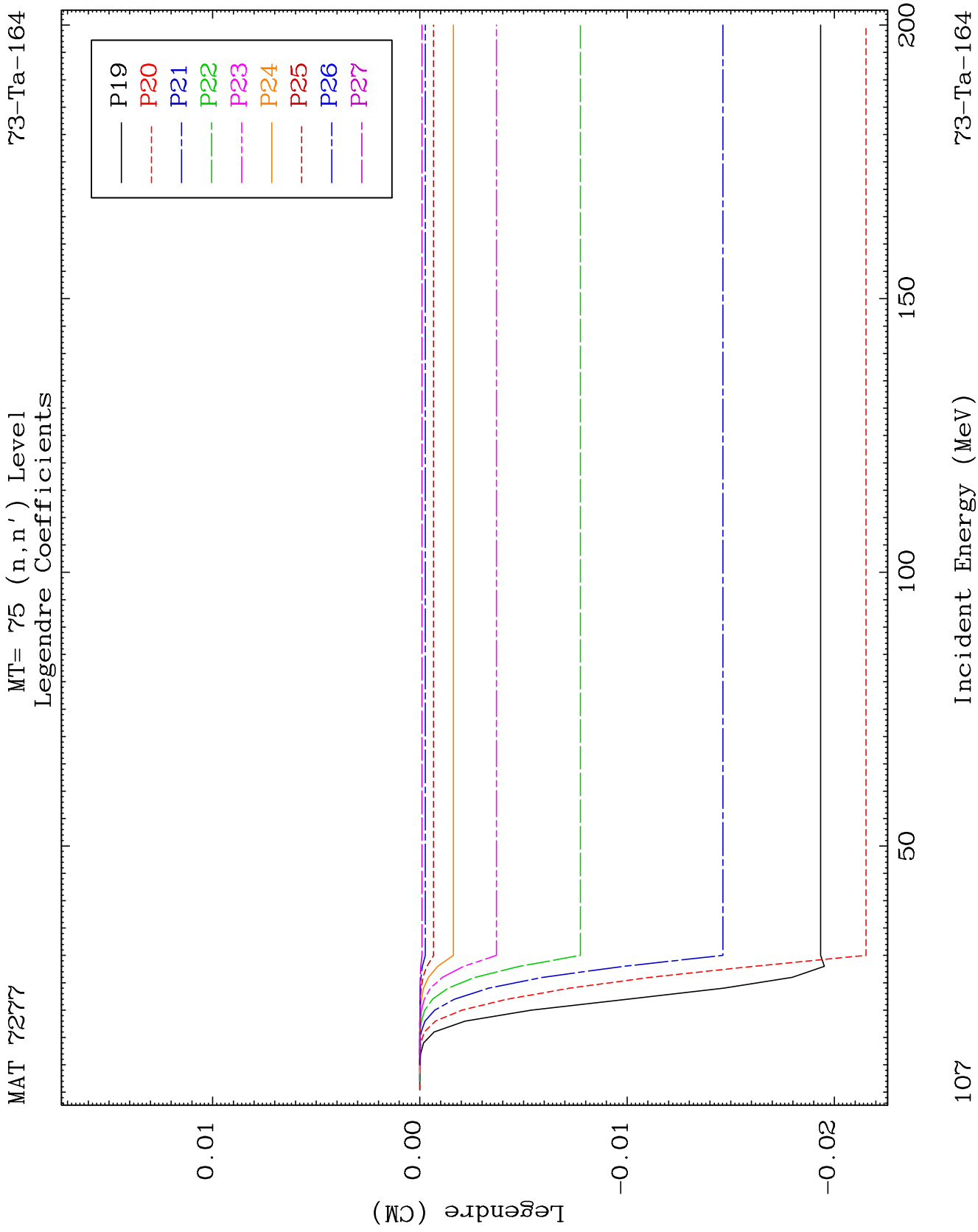


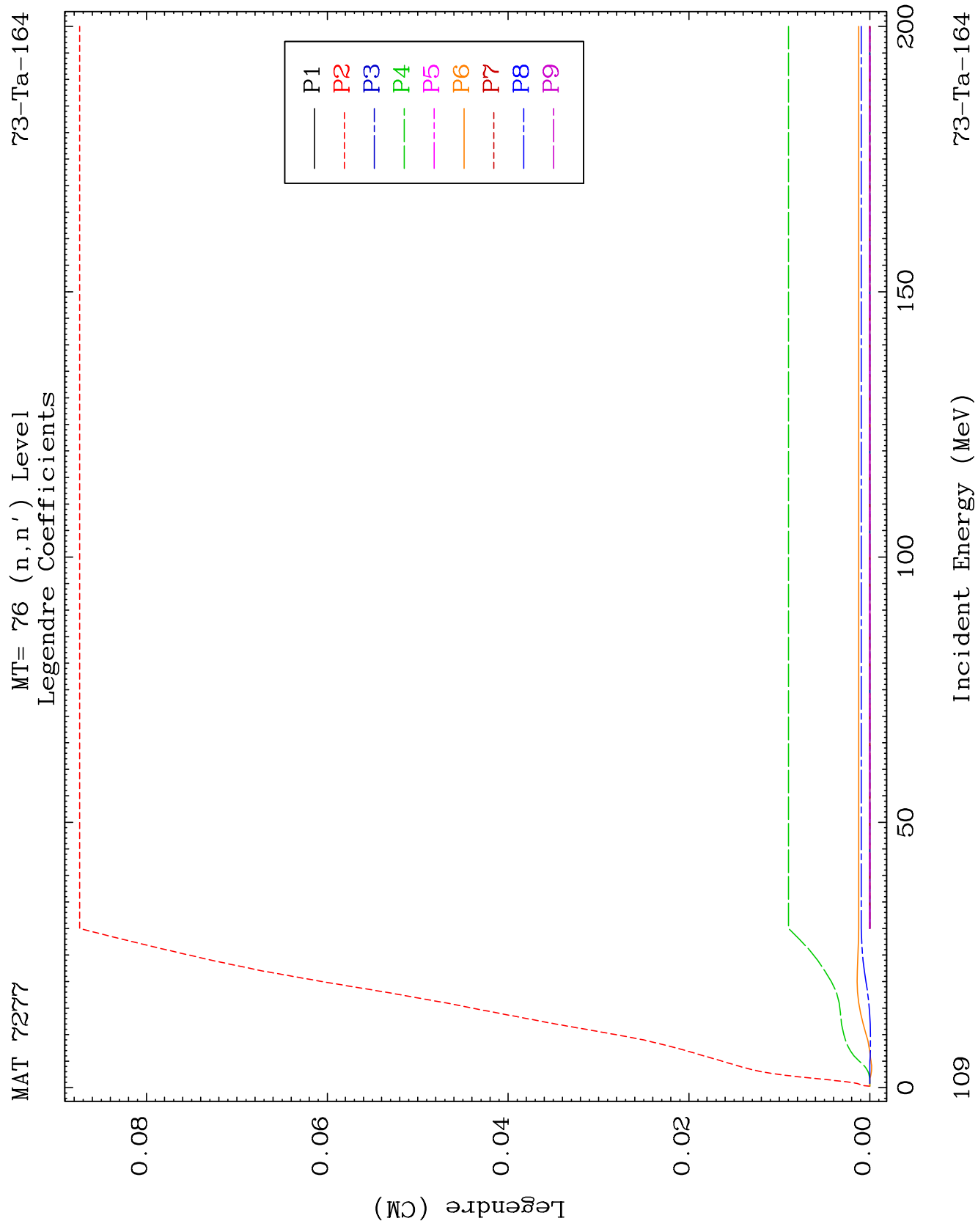
MAT 7277

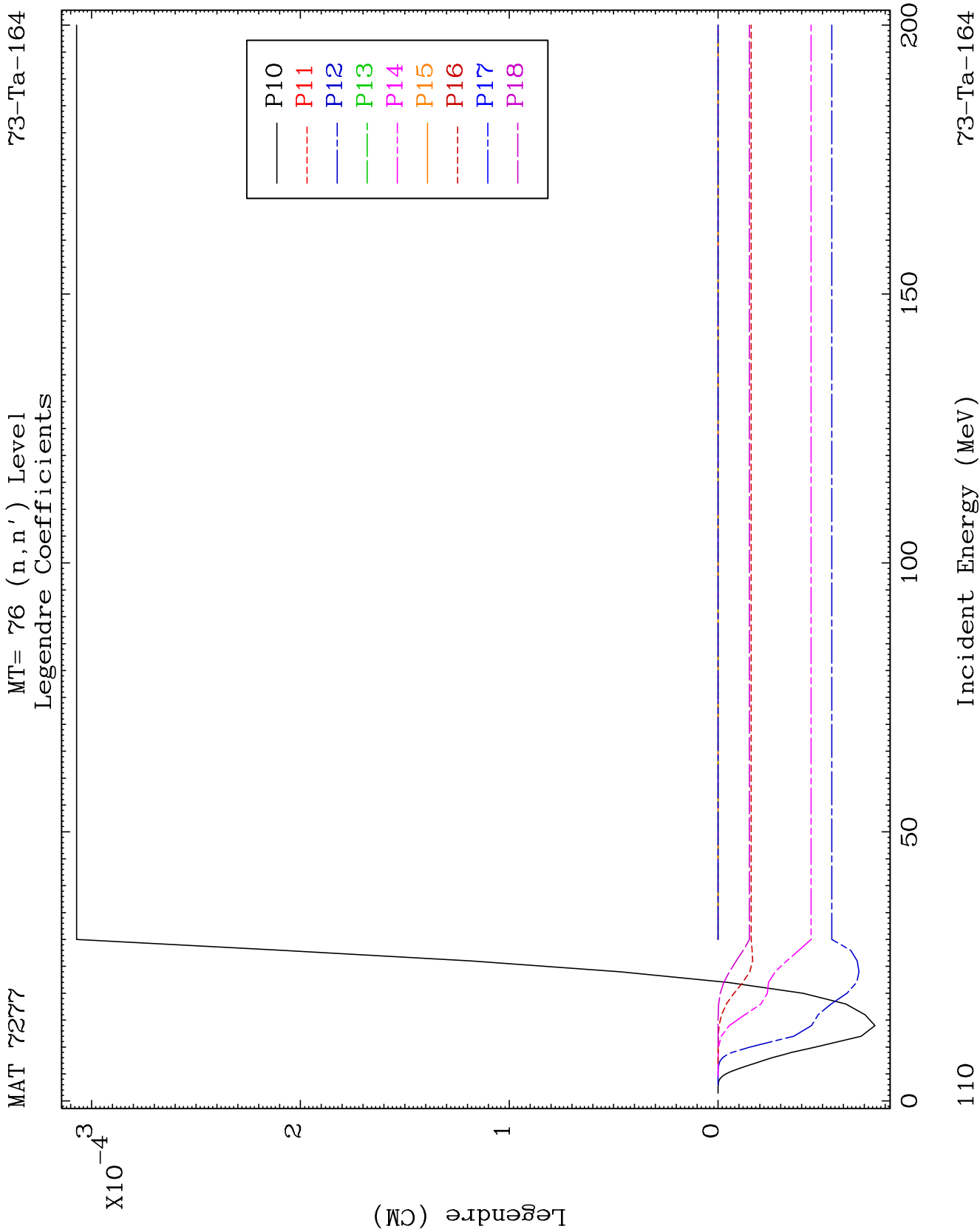
MT= 75 (n, n') Level
Legendre Coefficients

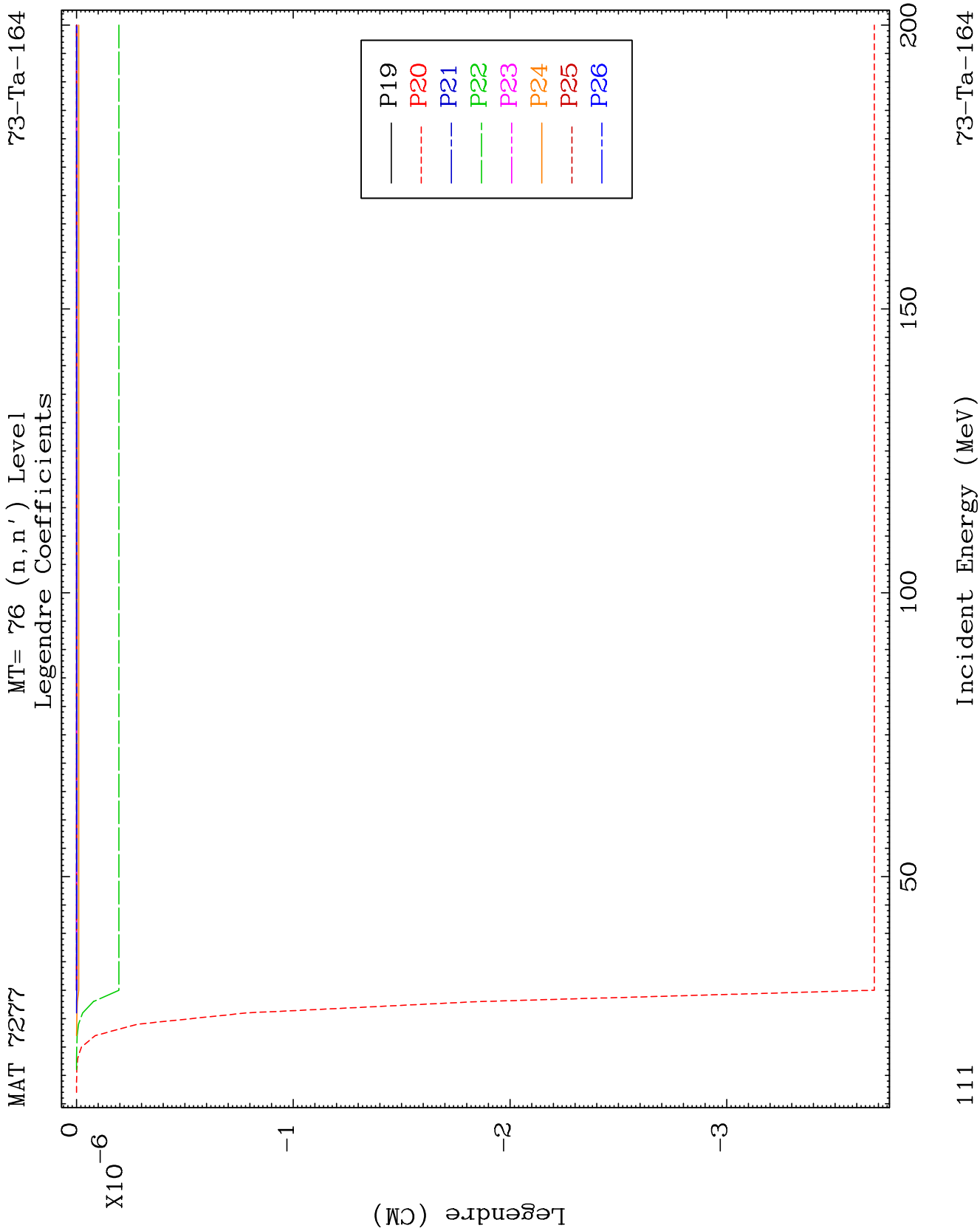
73-Ta-164







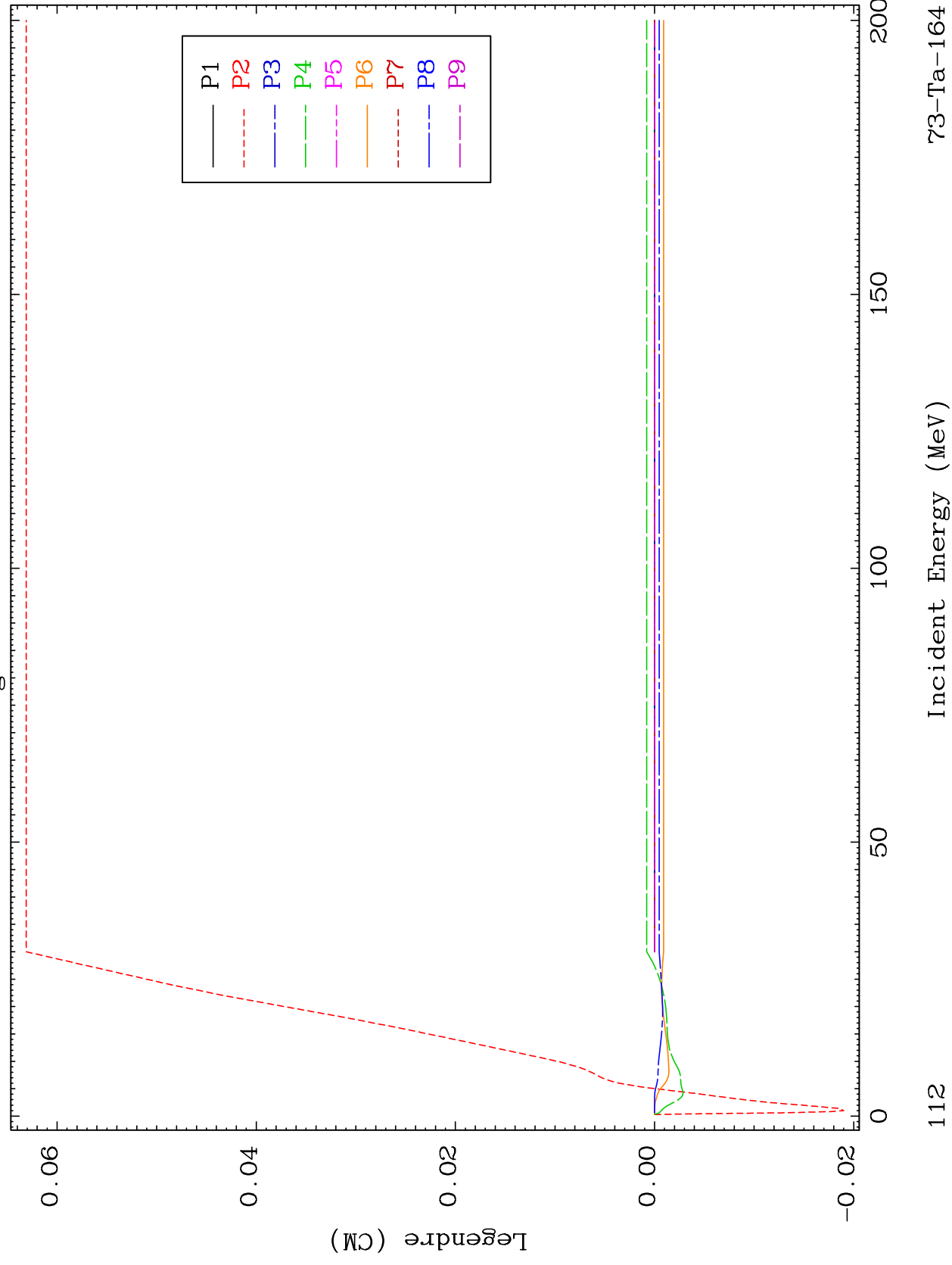


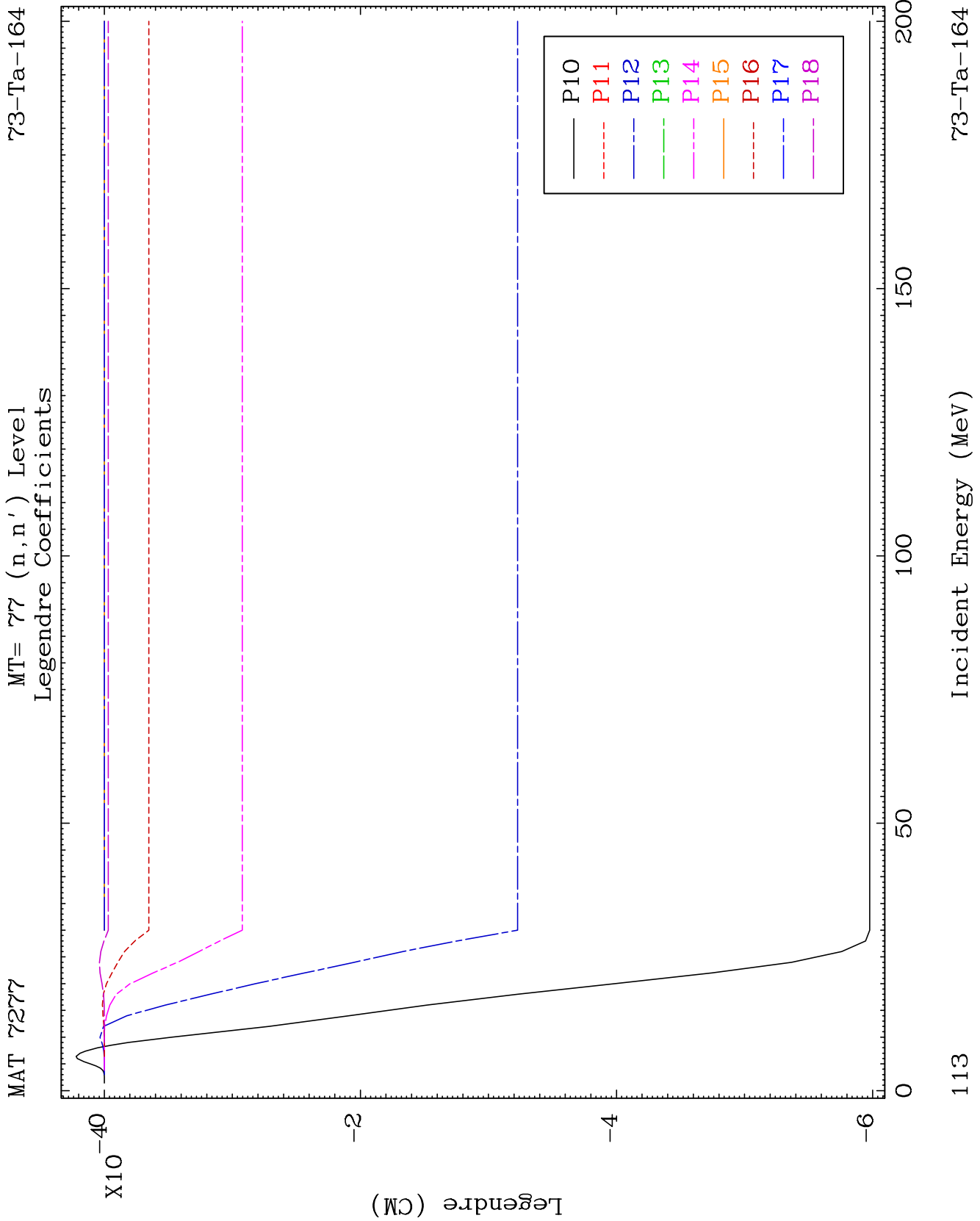


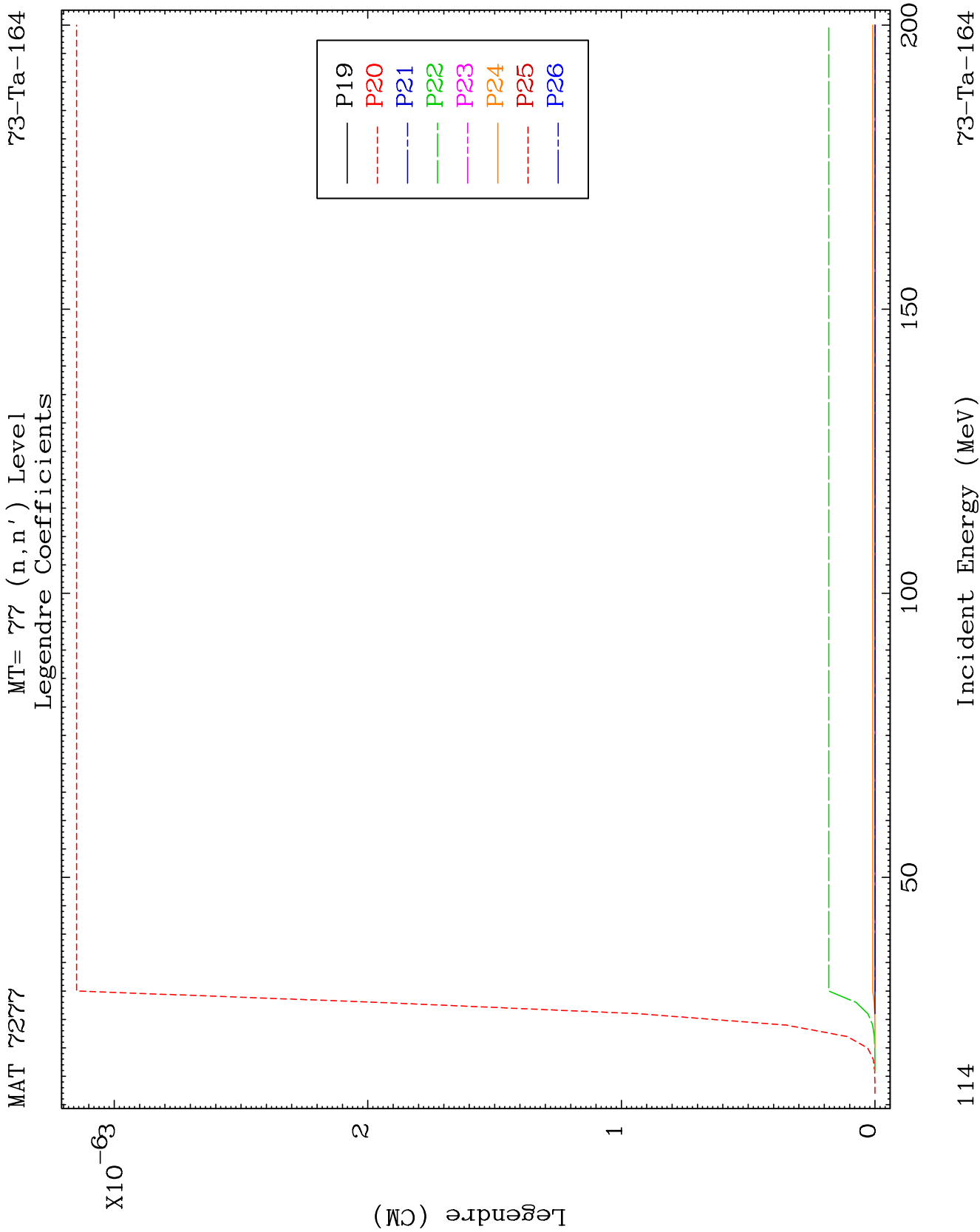
MAT 7277

MT= 77 (n, n') Level
Legendre Coefficients

73-Ta-164





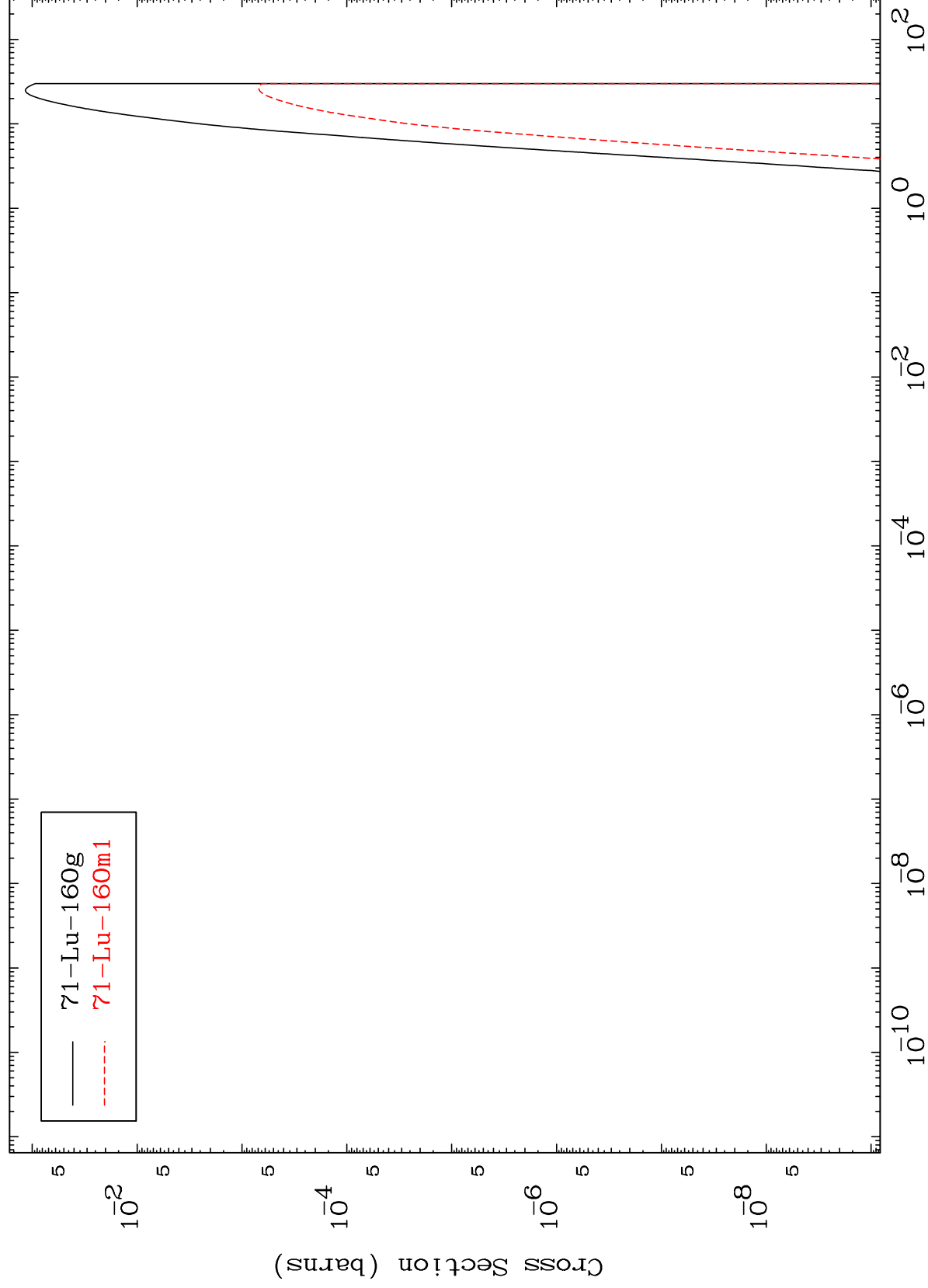


MAT 7277

$(n,n') \alpha$

$^{73}\text{Ta-164}$

Radionuclide Production Cross Section

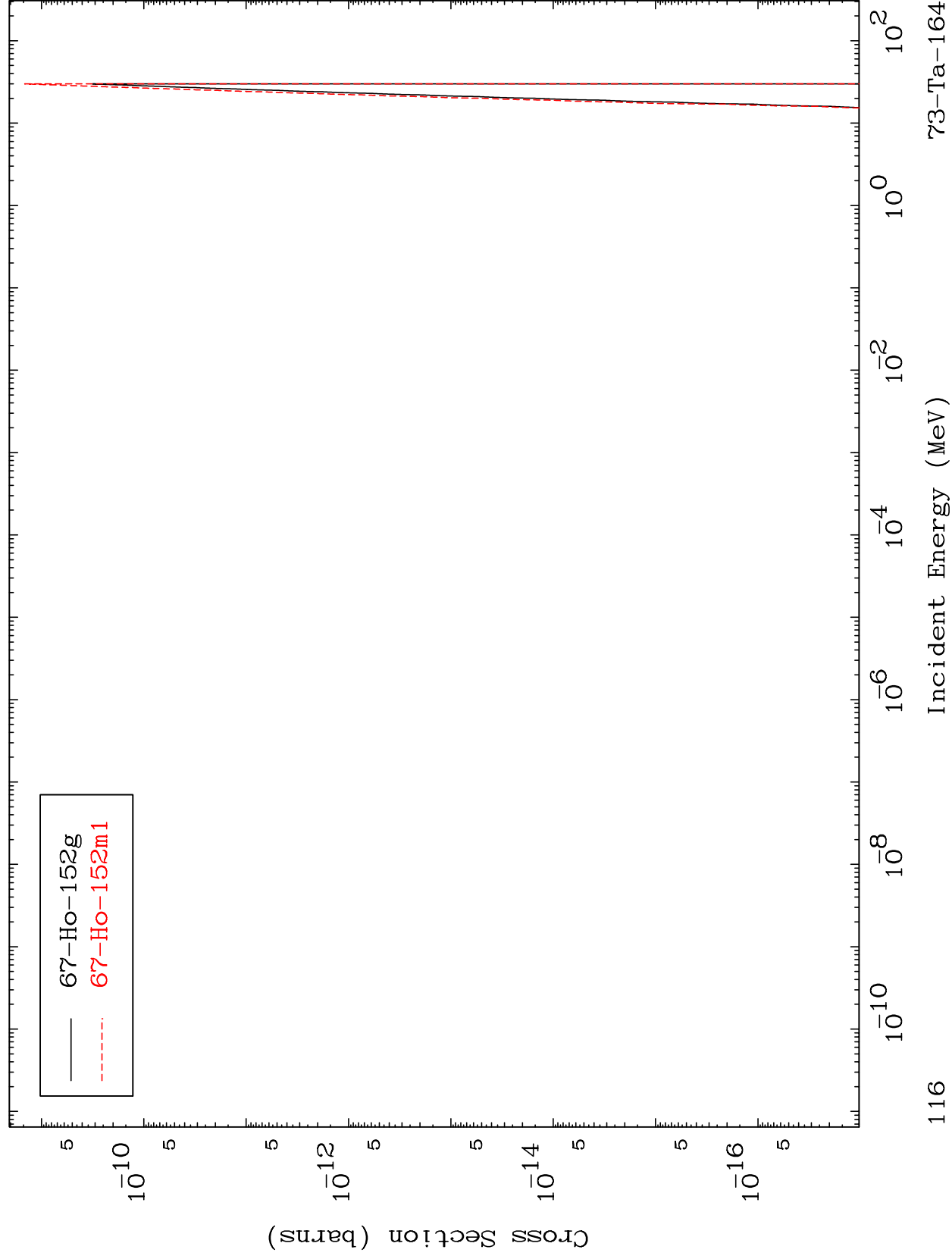


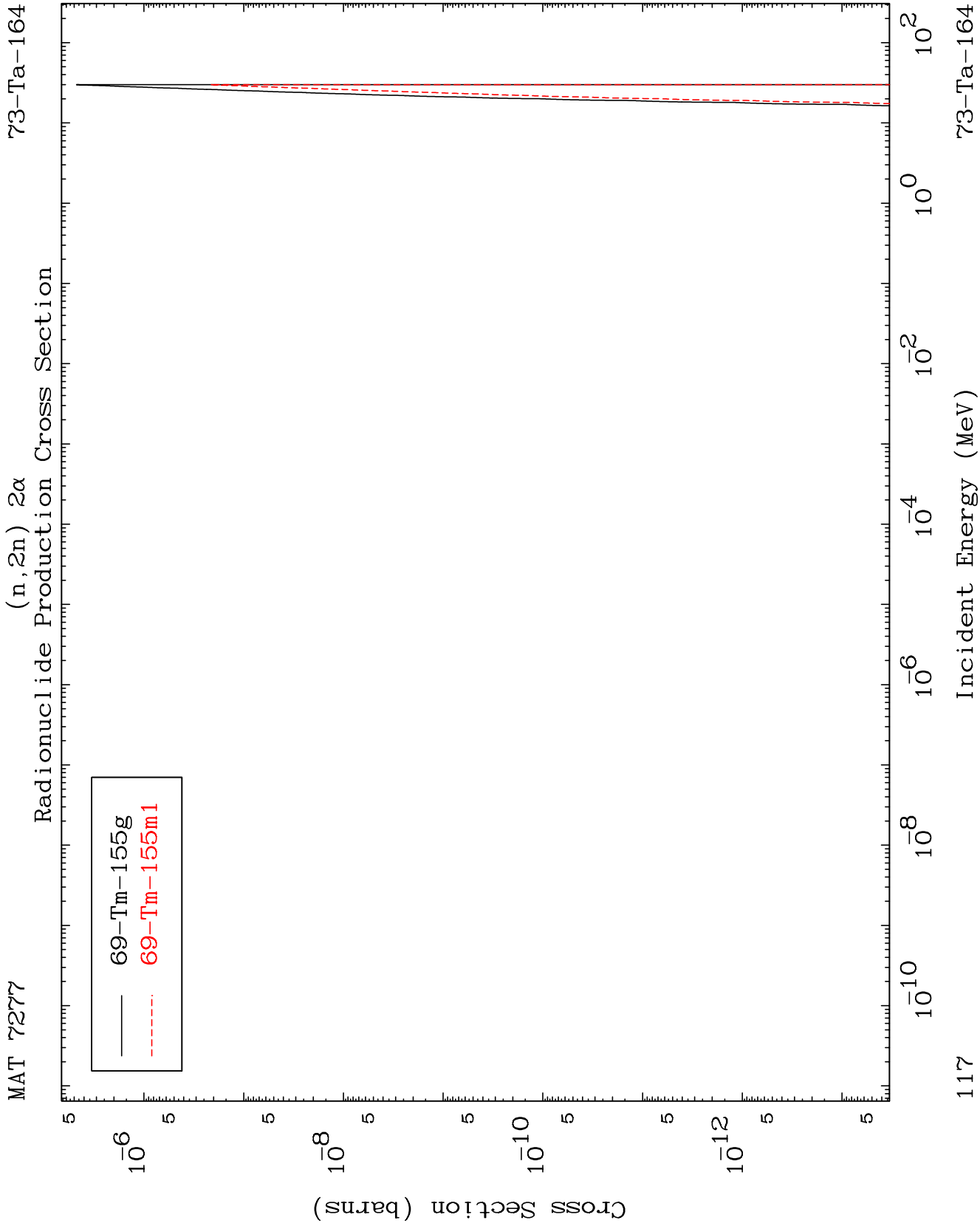
115

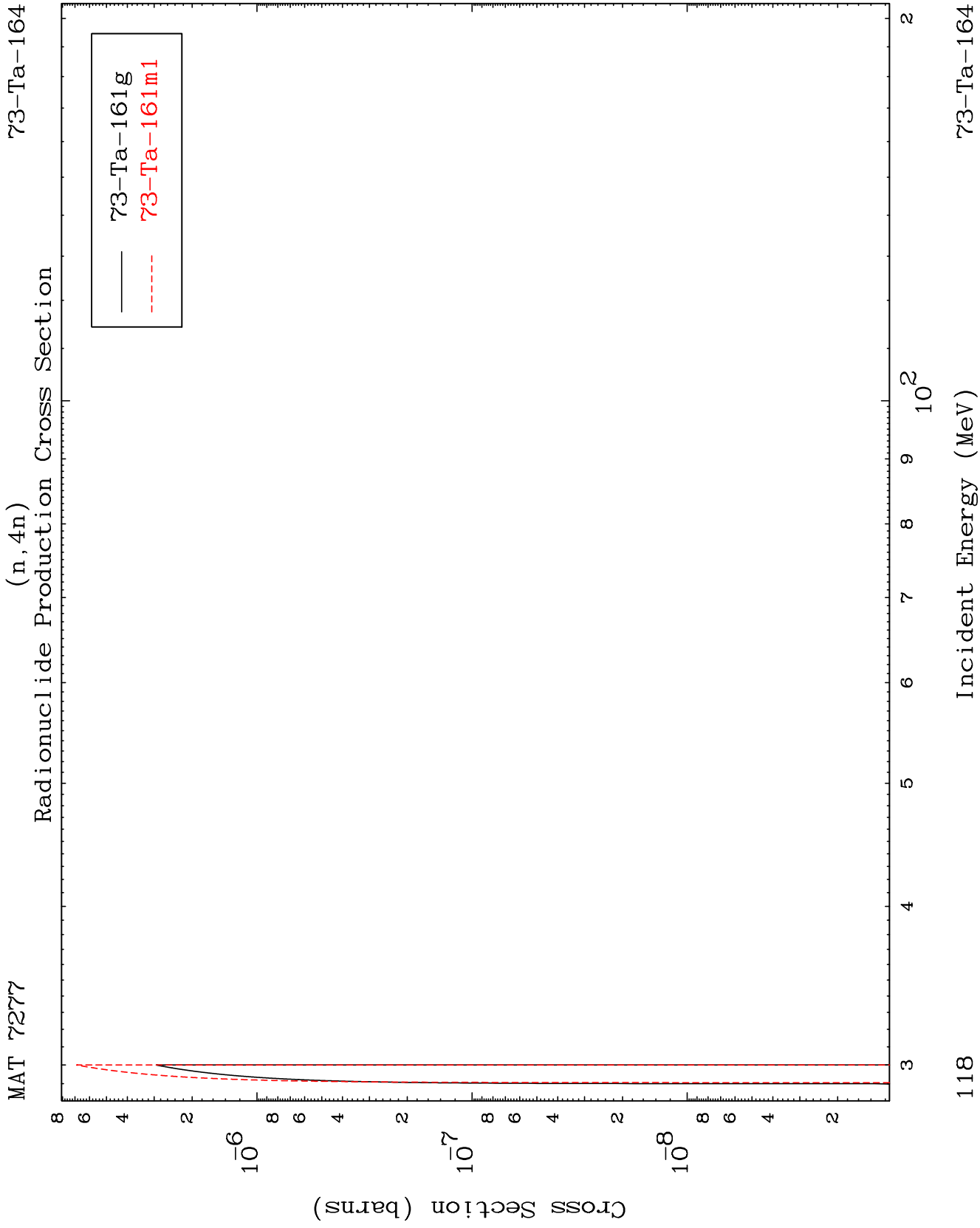
Incident Energy (MeV)

$^{73}\text{Ta-164}$

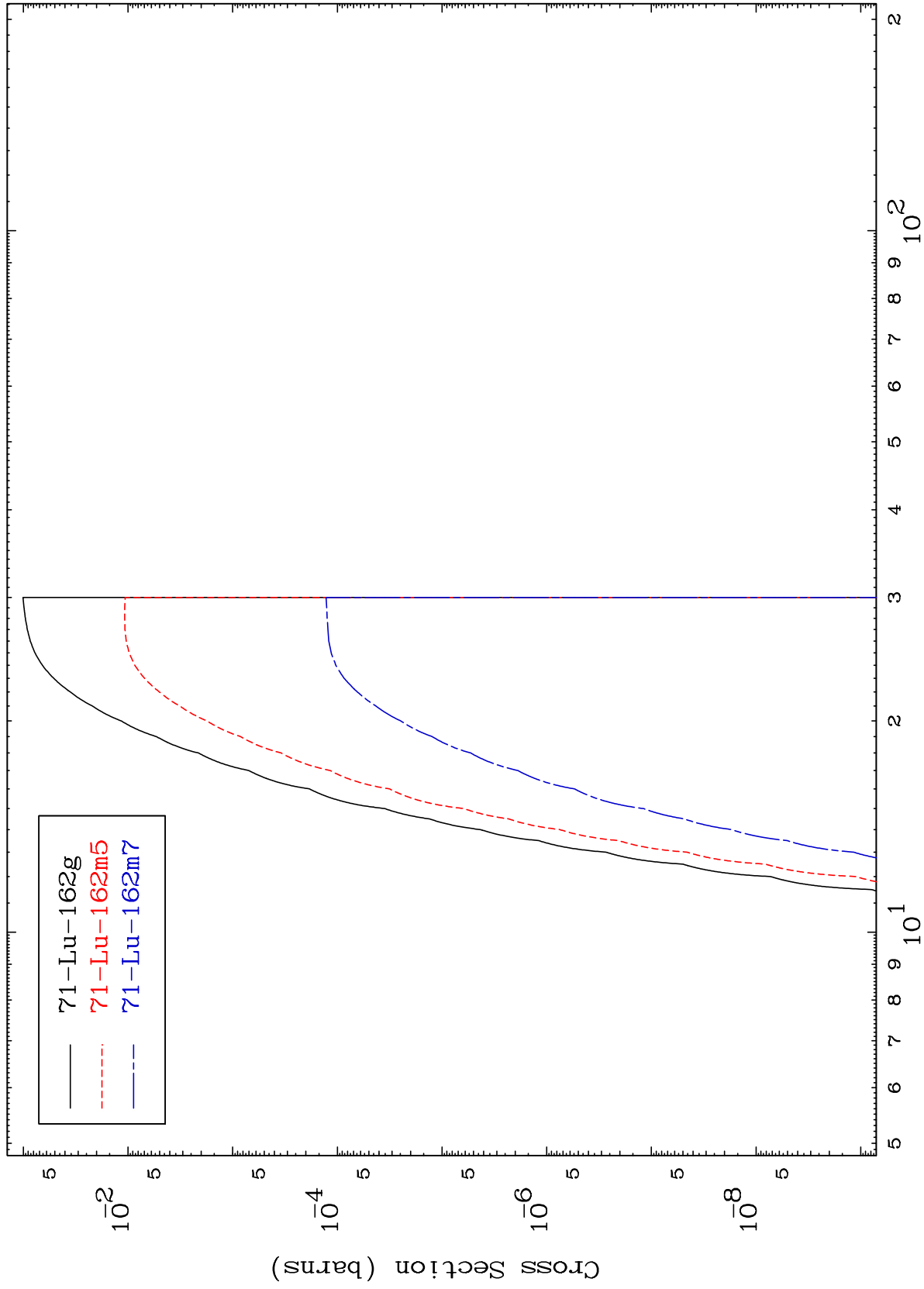
Radionuclide Production Cross Section

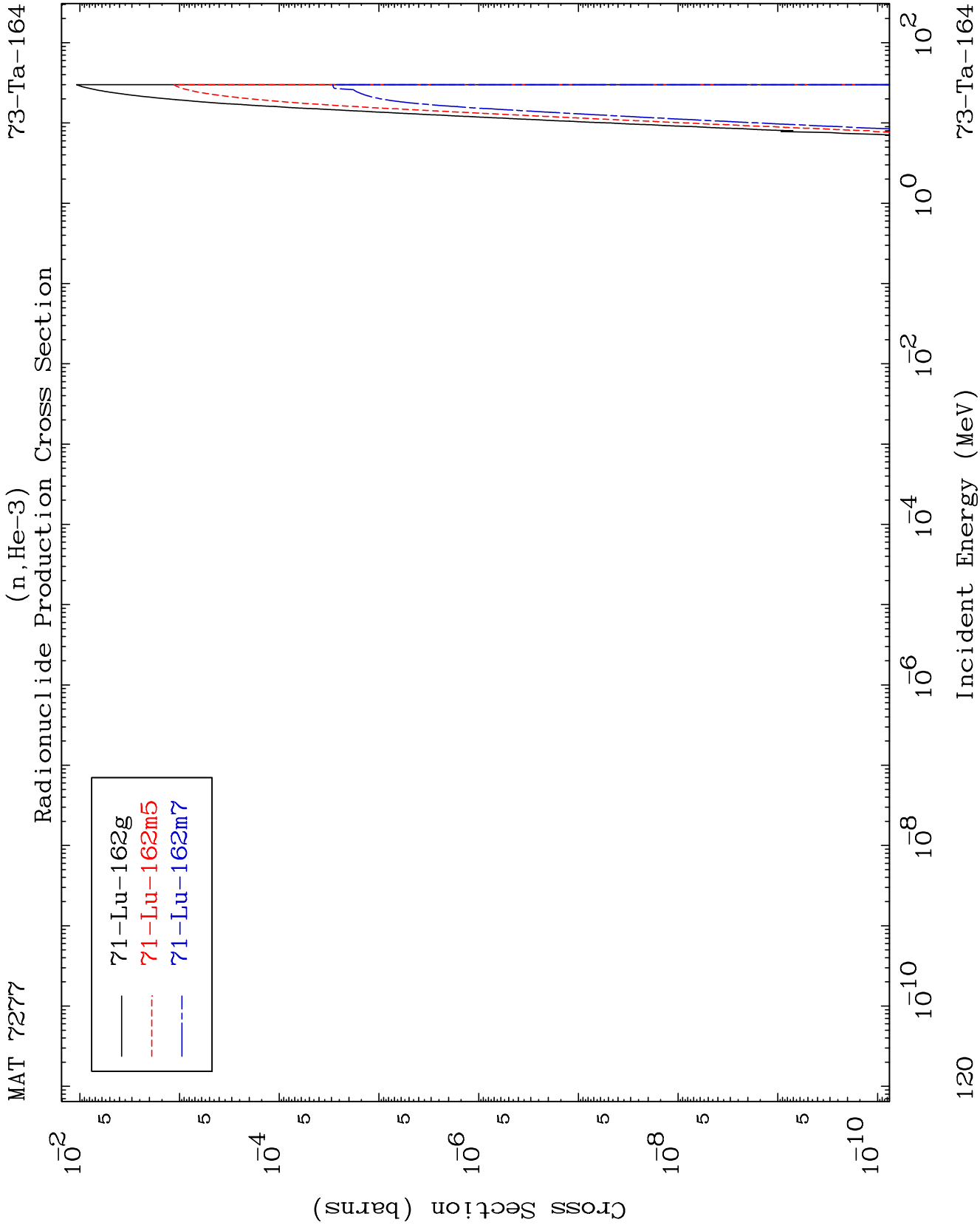






Radionuclide Production Cross Section



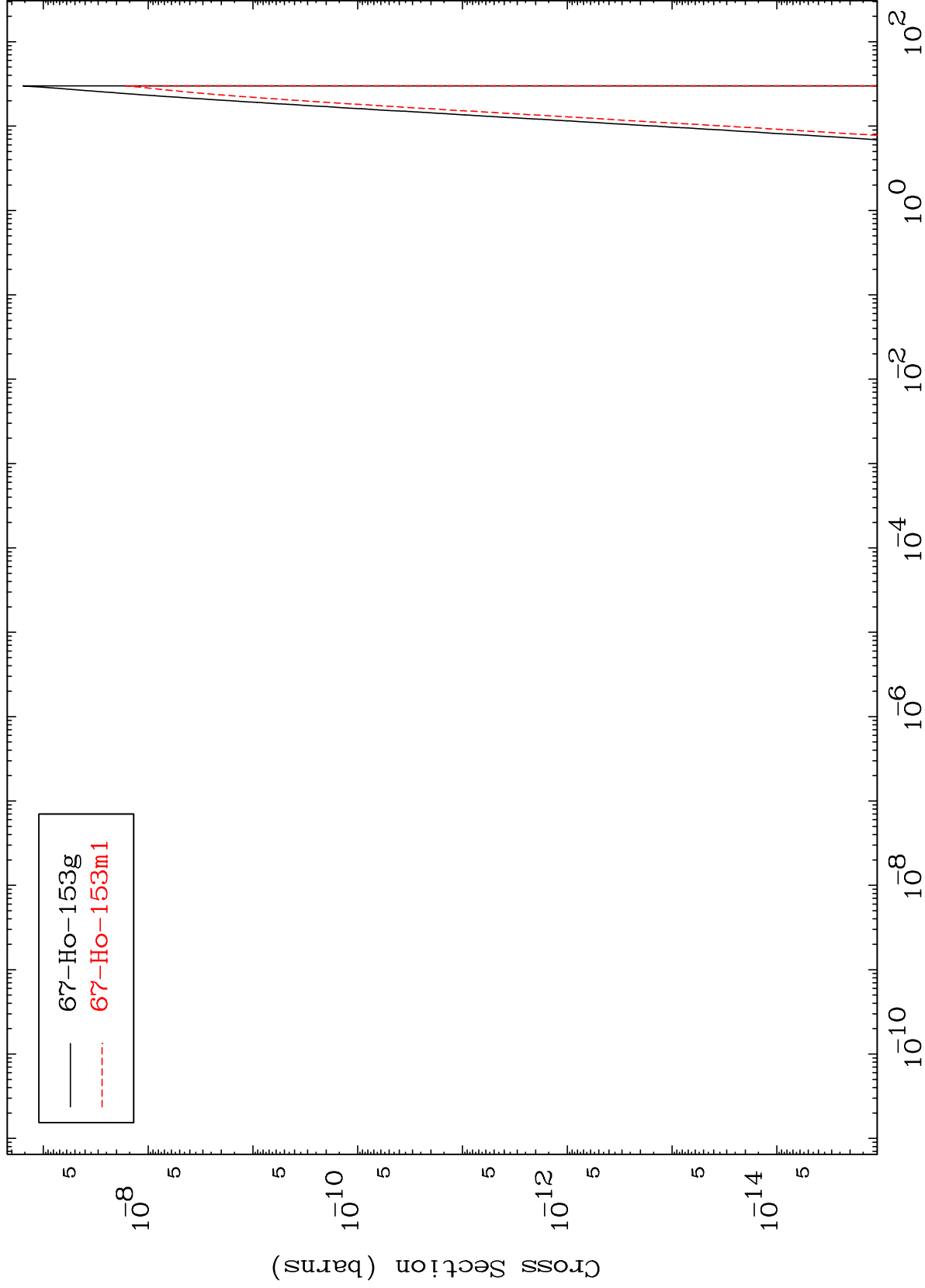


MAT 7277

(n,3α)

73-Ta-164

Radionuclide Production Cross Section



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Incident Energy (MeV)

73-Ta-164

