

Program EVALPLOT  
(Version 2021-1)

by

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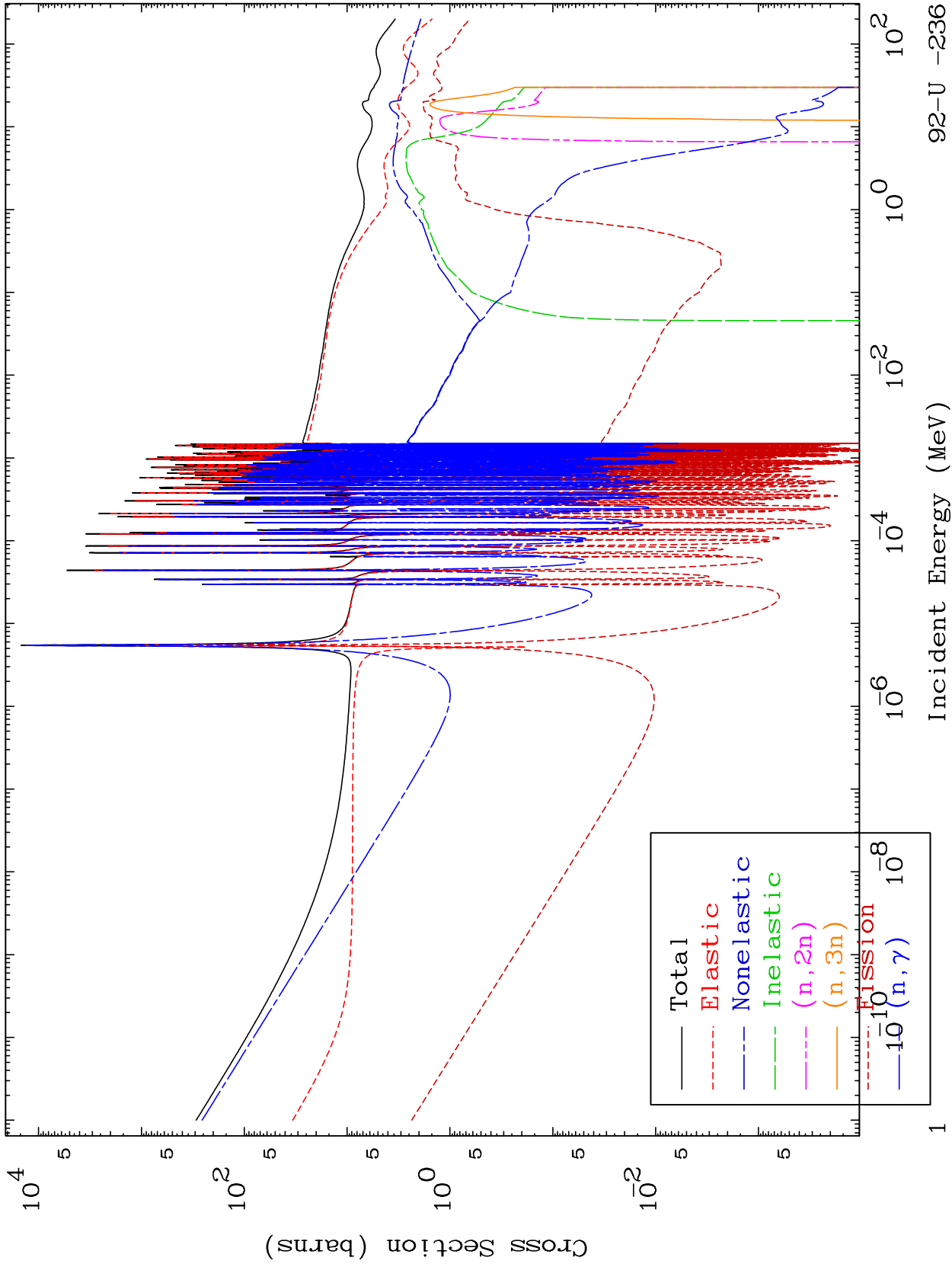
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Press Mouse Button to Start

MAT 9231

Neutron Major  
293 Kelvin Cross Sections

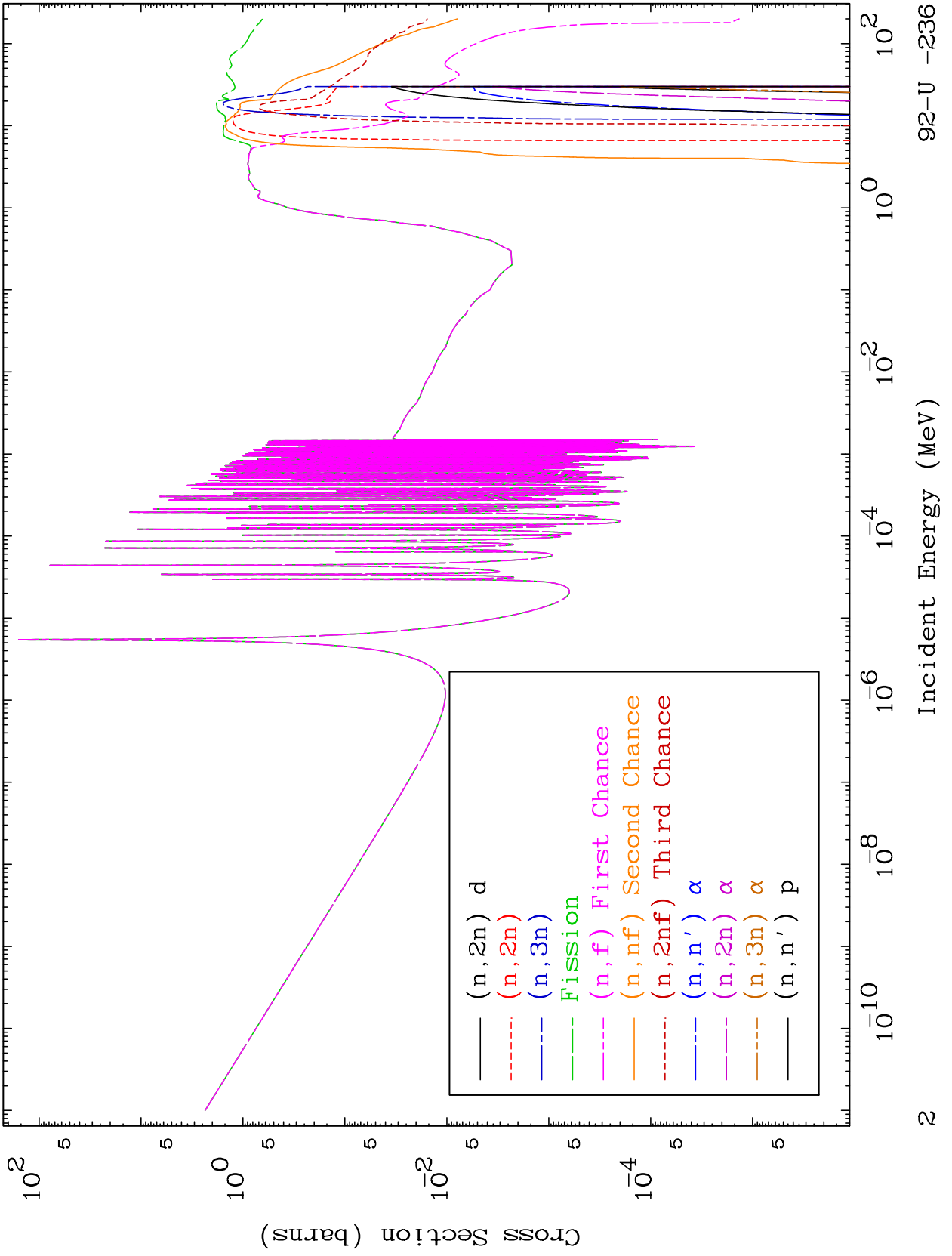
92-U -236

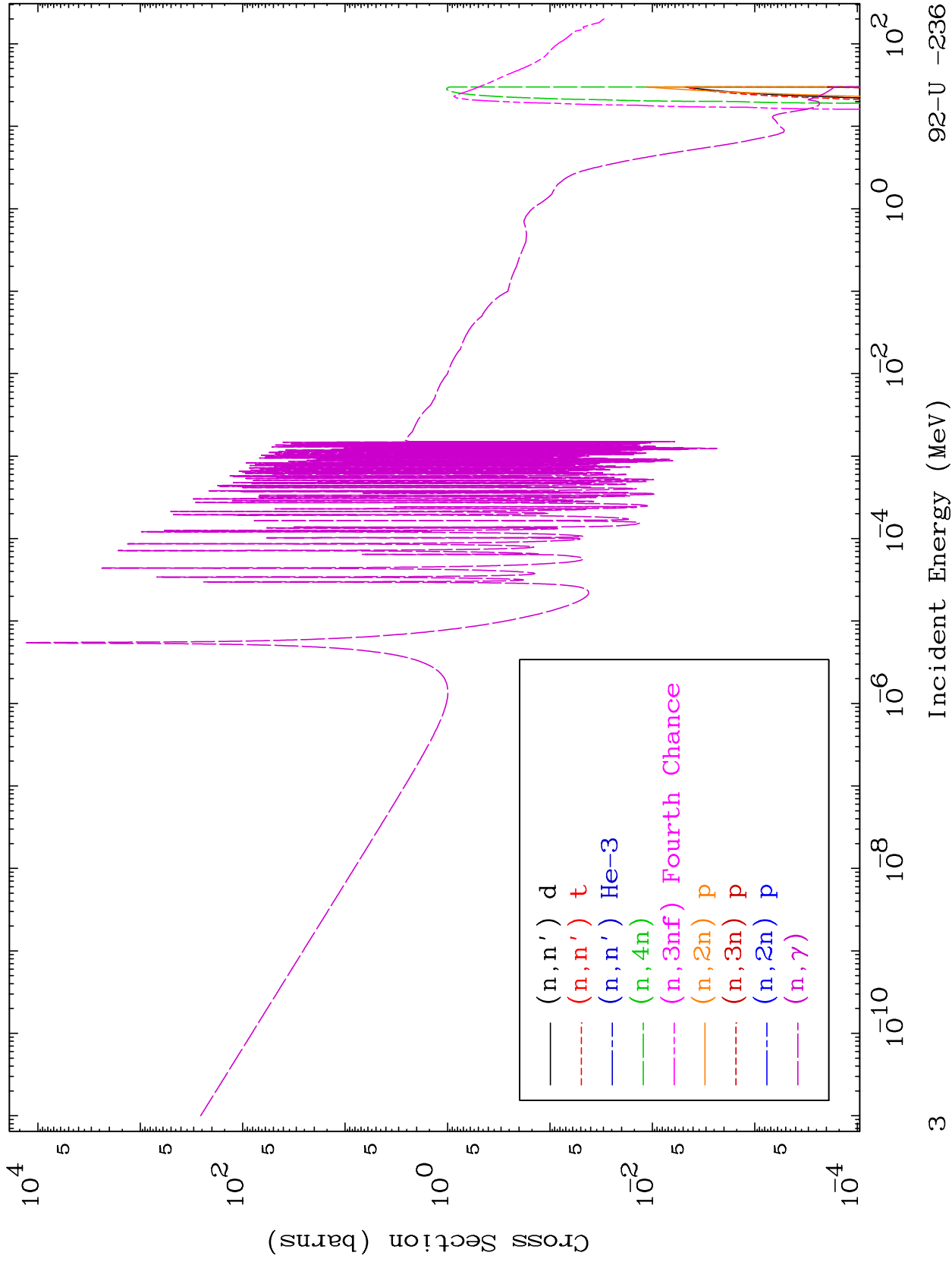


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Neutron Absorption  
293 Kelvin Cross Sections

92-U -236

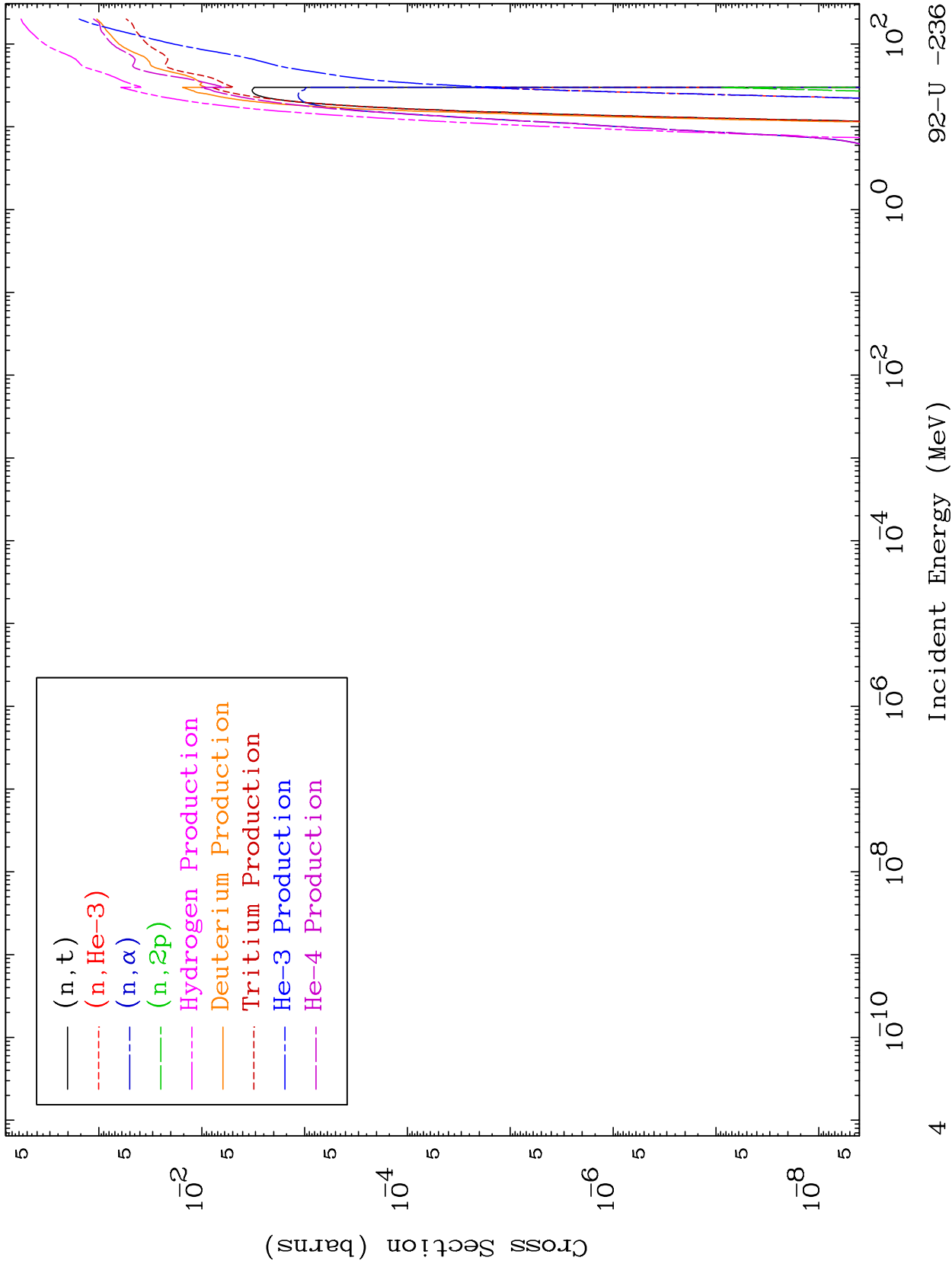




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Neutron Absorption  
293 Kelvin Cross Sections

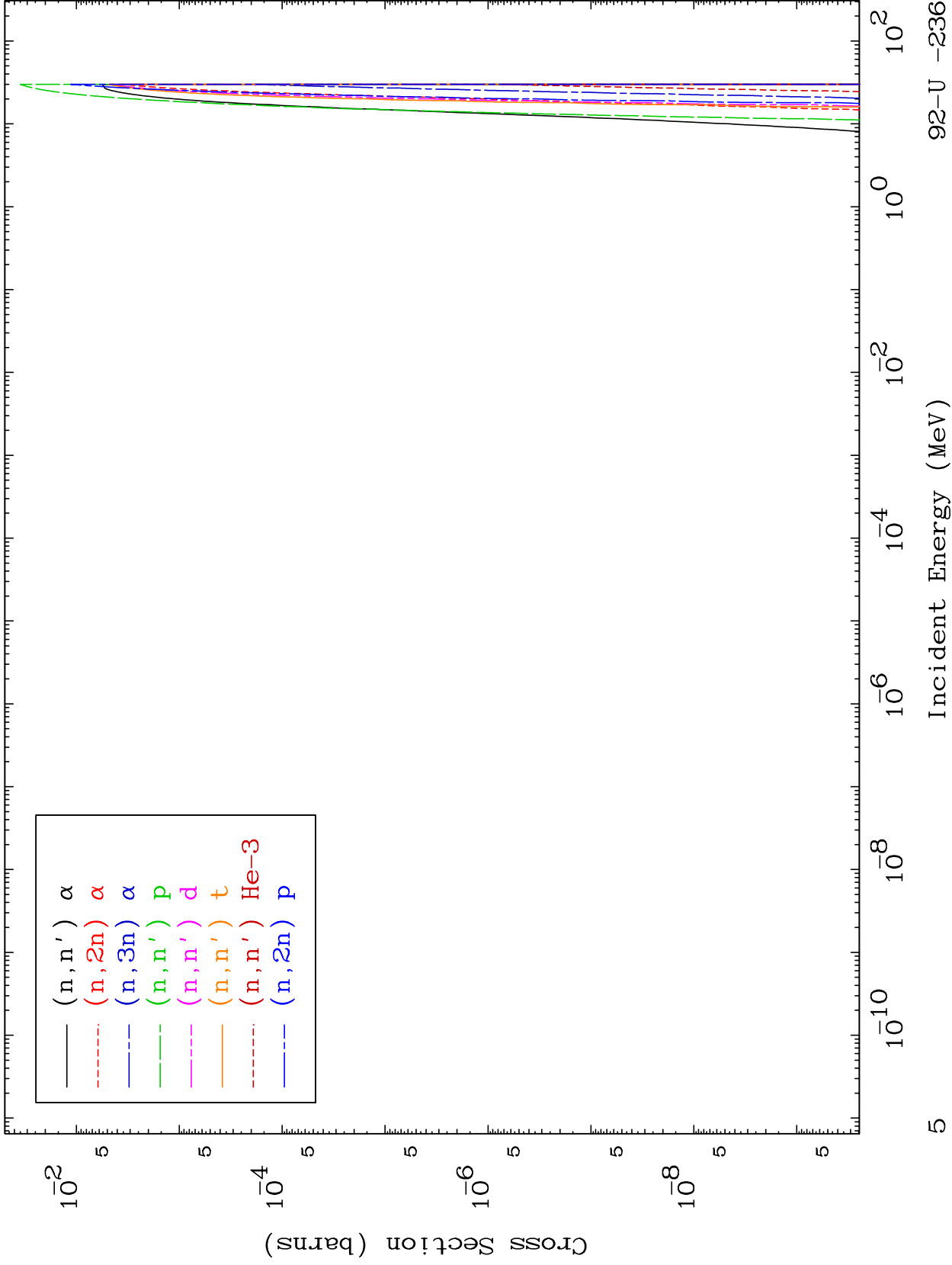
92-U -236



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Charged Particle  
293 Kelvin Cross Sections

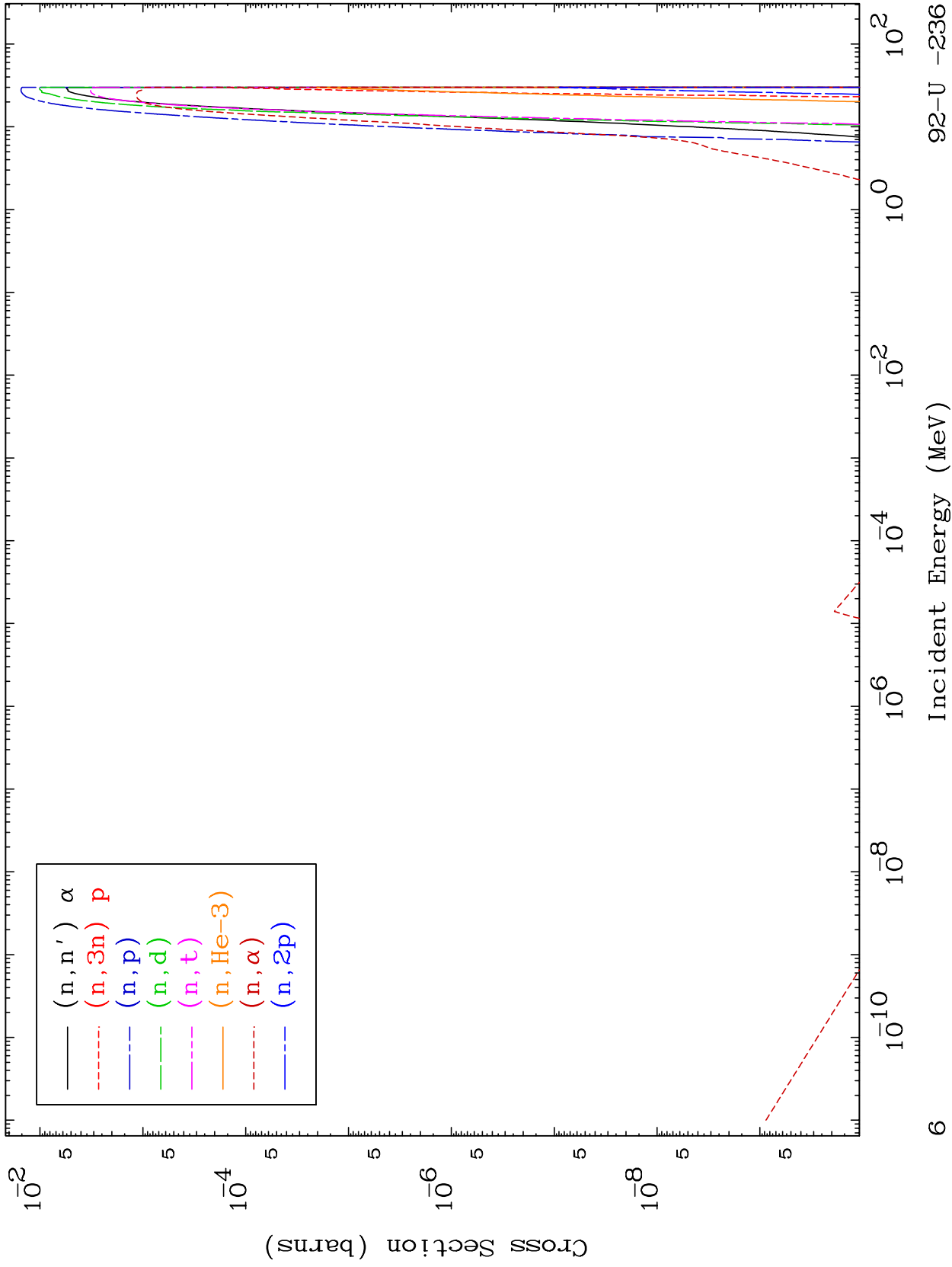
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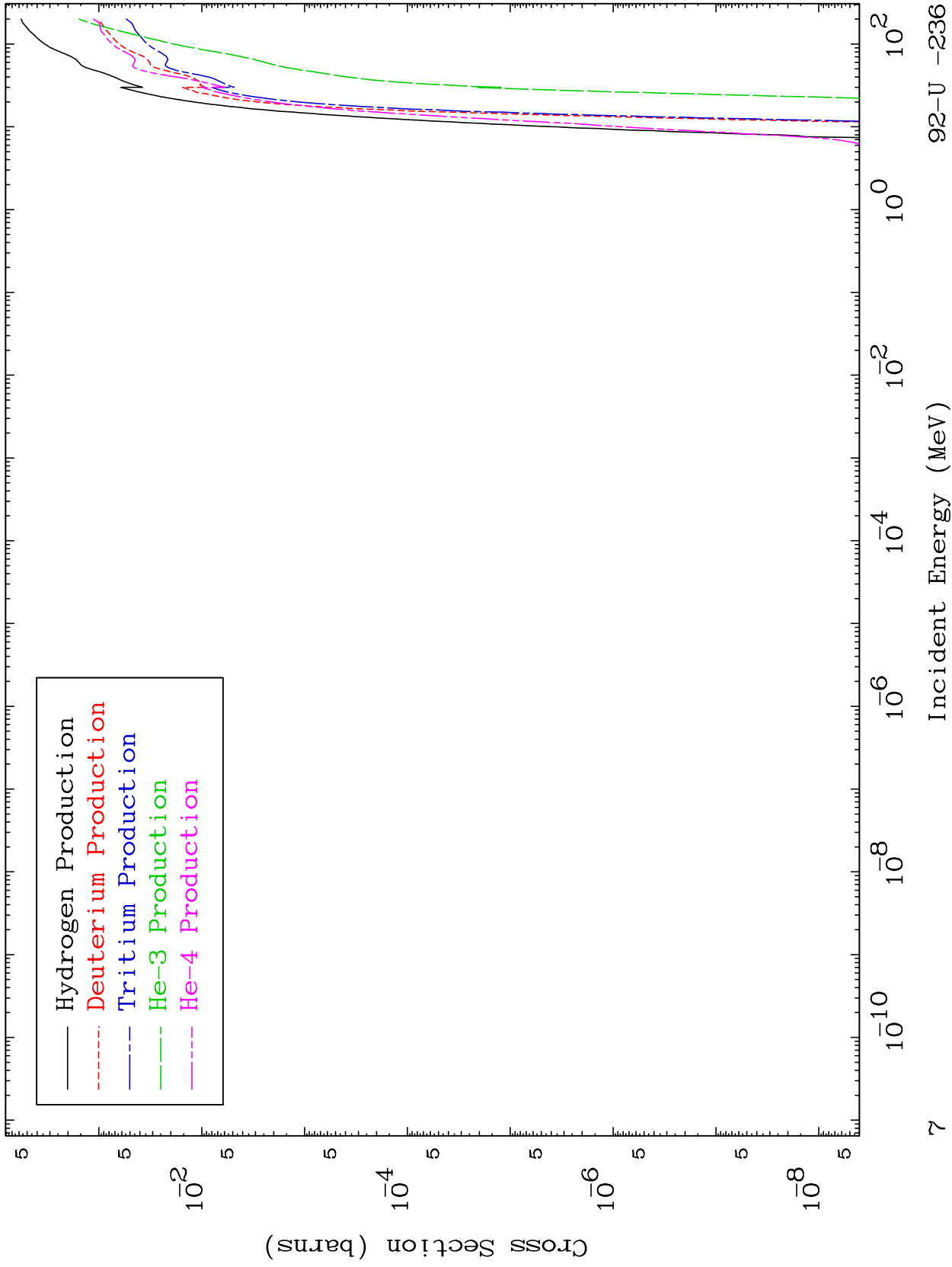


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Charged Particle  
293 Kelvin Cross Sections

92-U -236

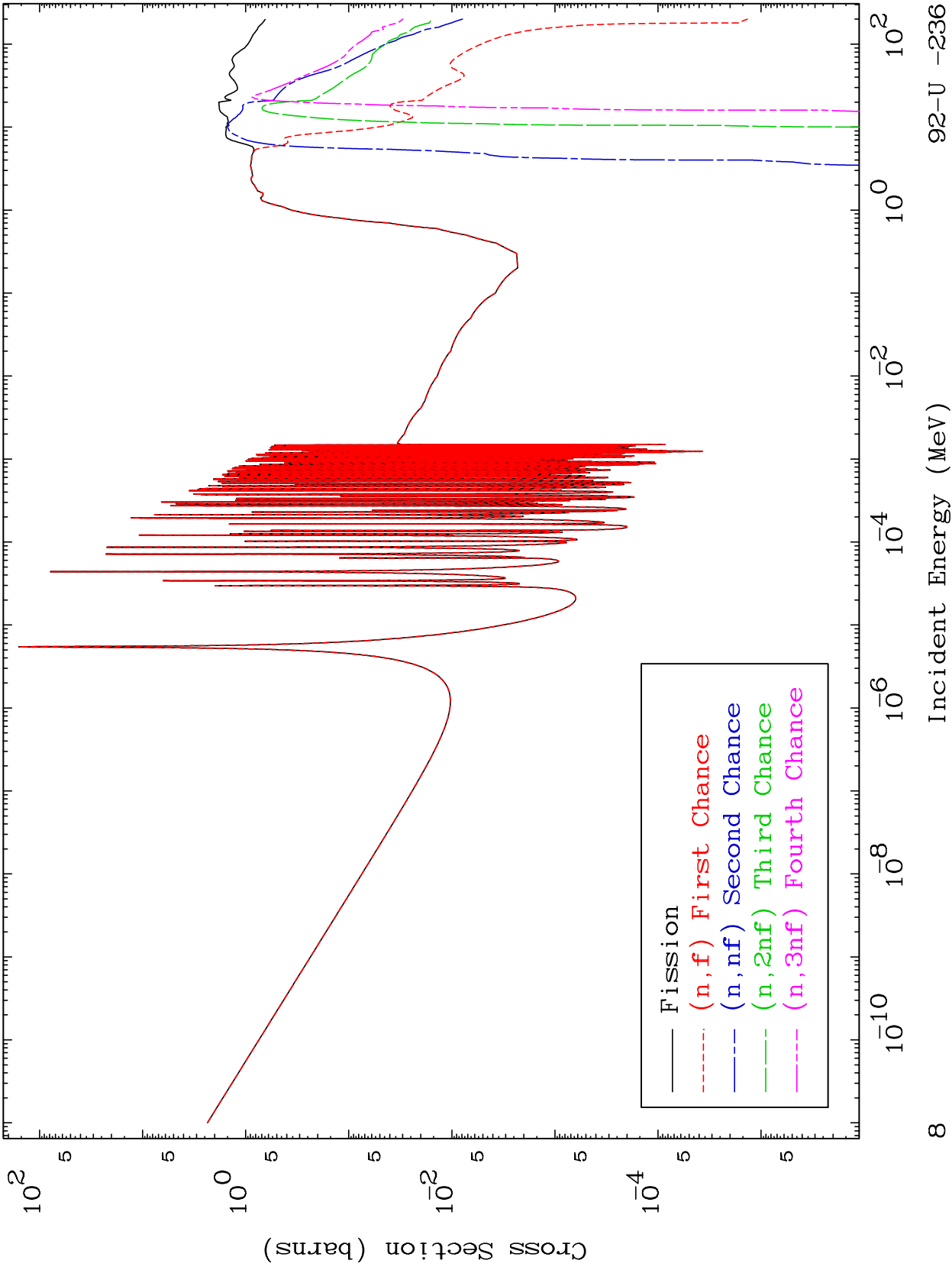


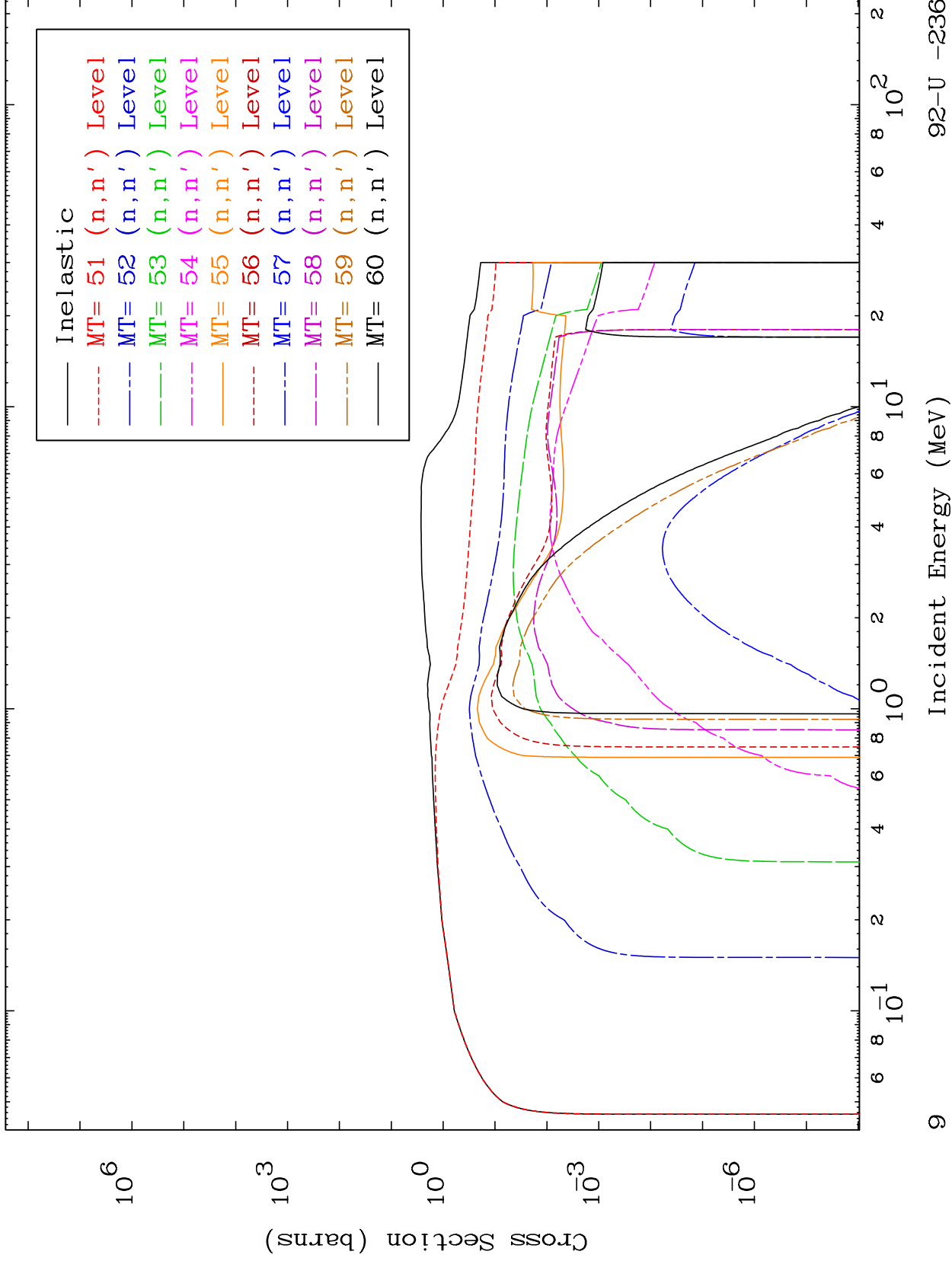


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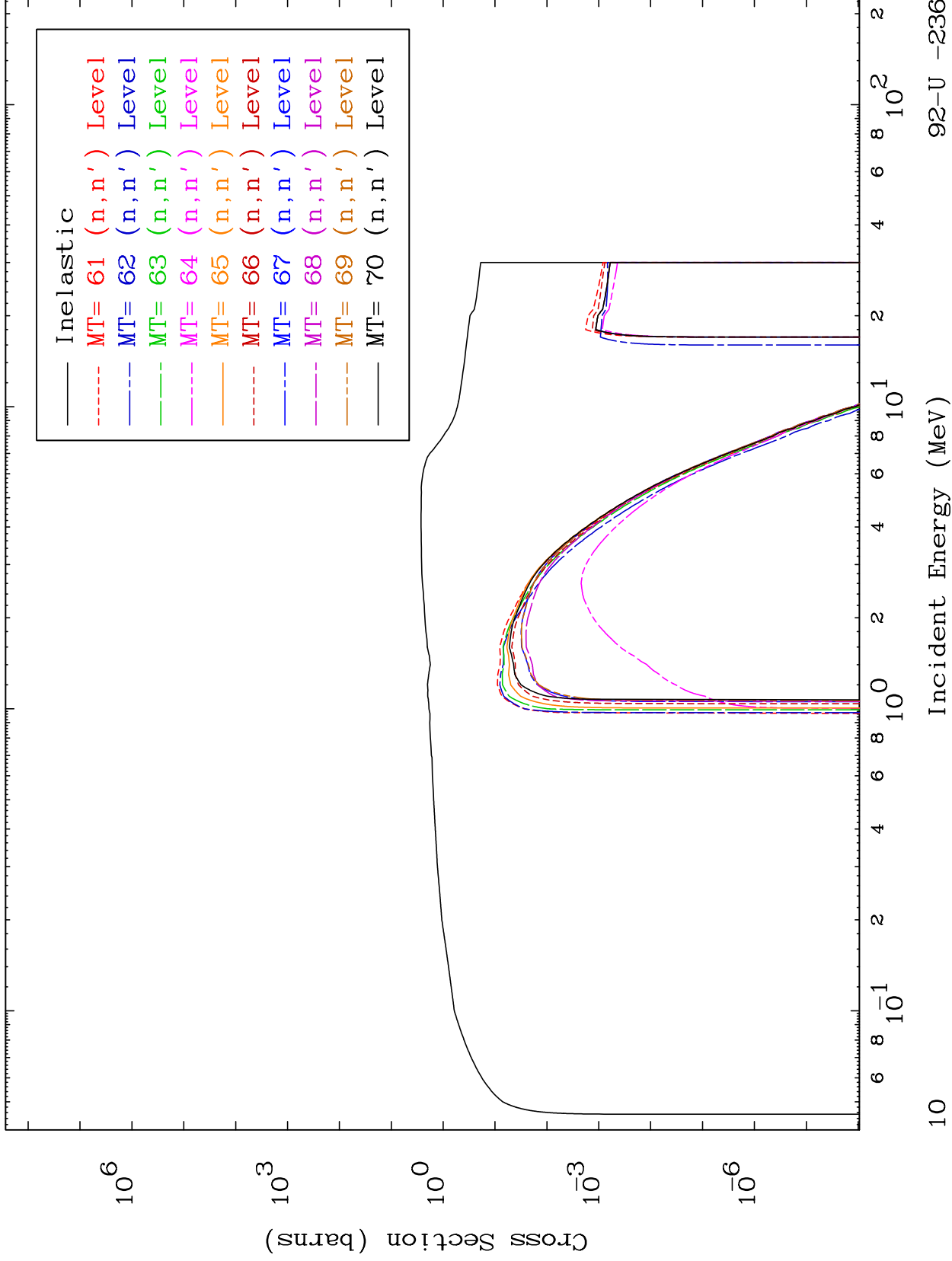
Fission  
293 Kelvin Cross Sections

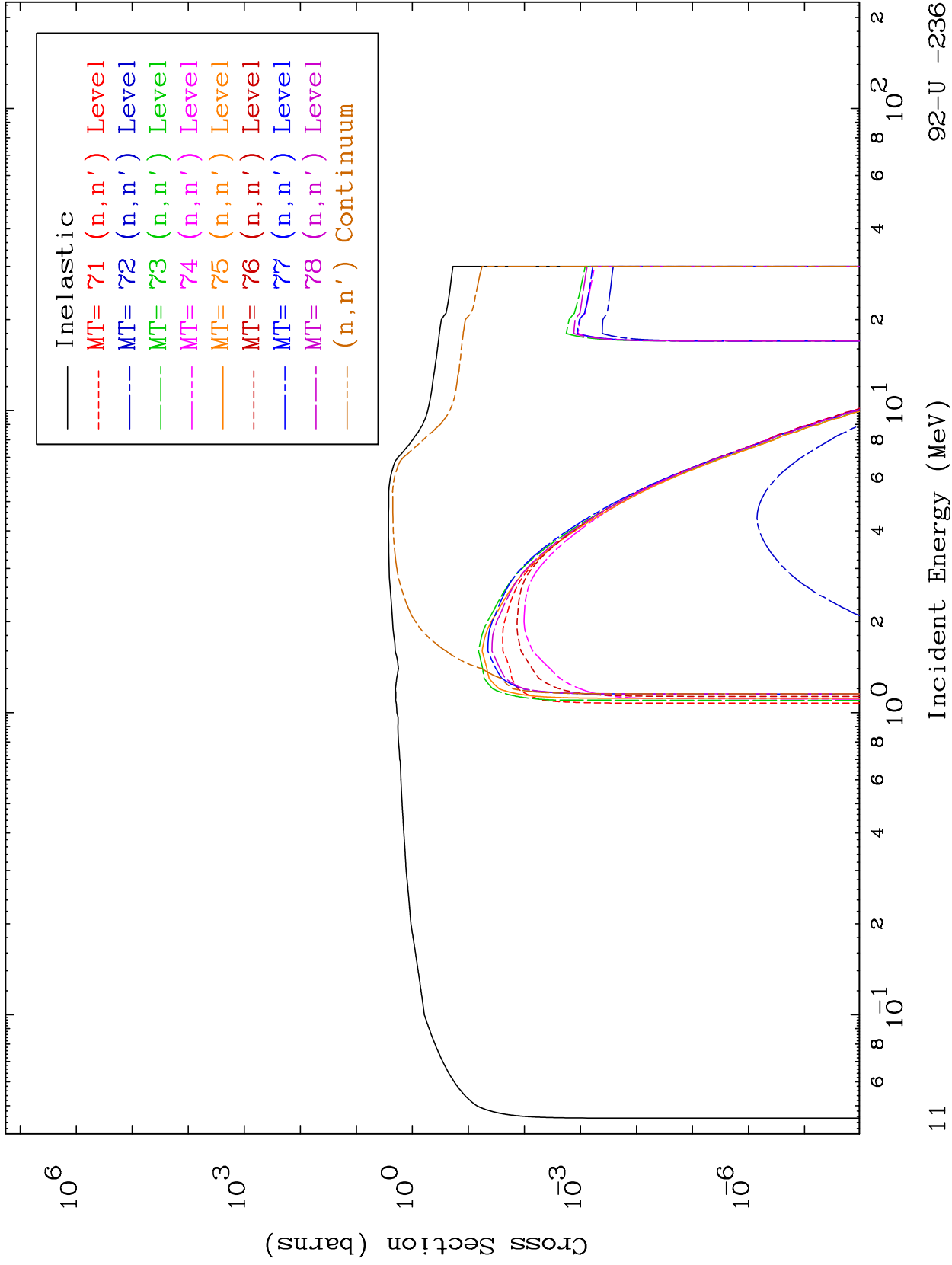
92-U -236





293 Kelvin Cross Sections

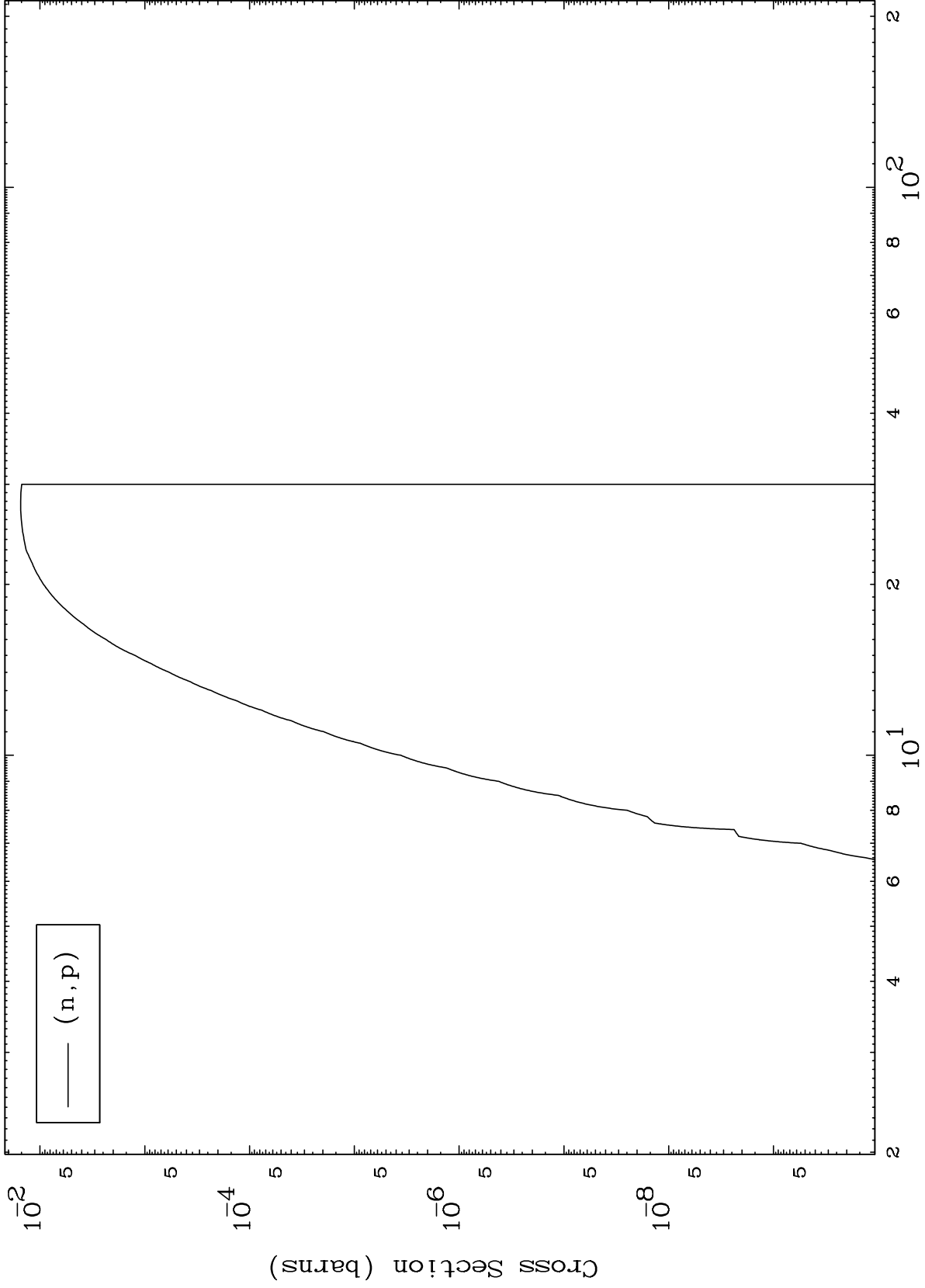




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(n,p) Levels  
293 Kelvin Cross Sections

92-U -236



(n,p)

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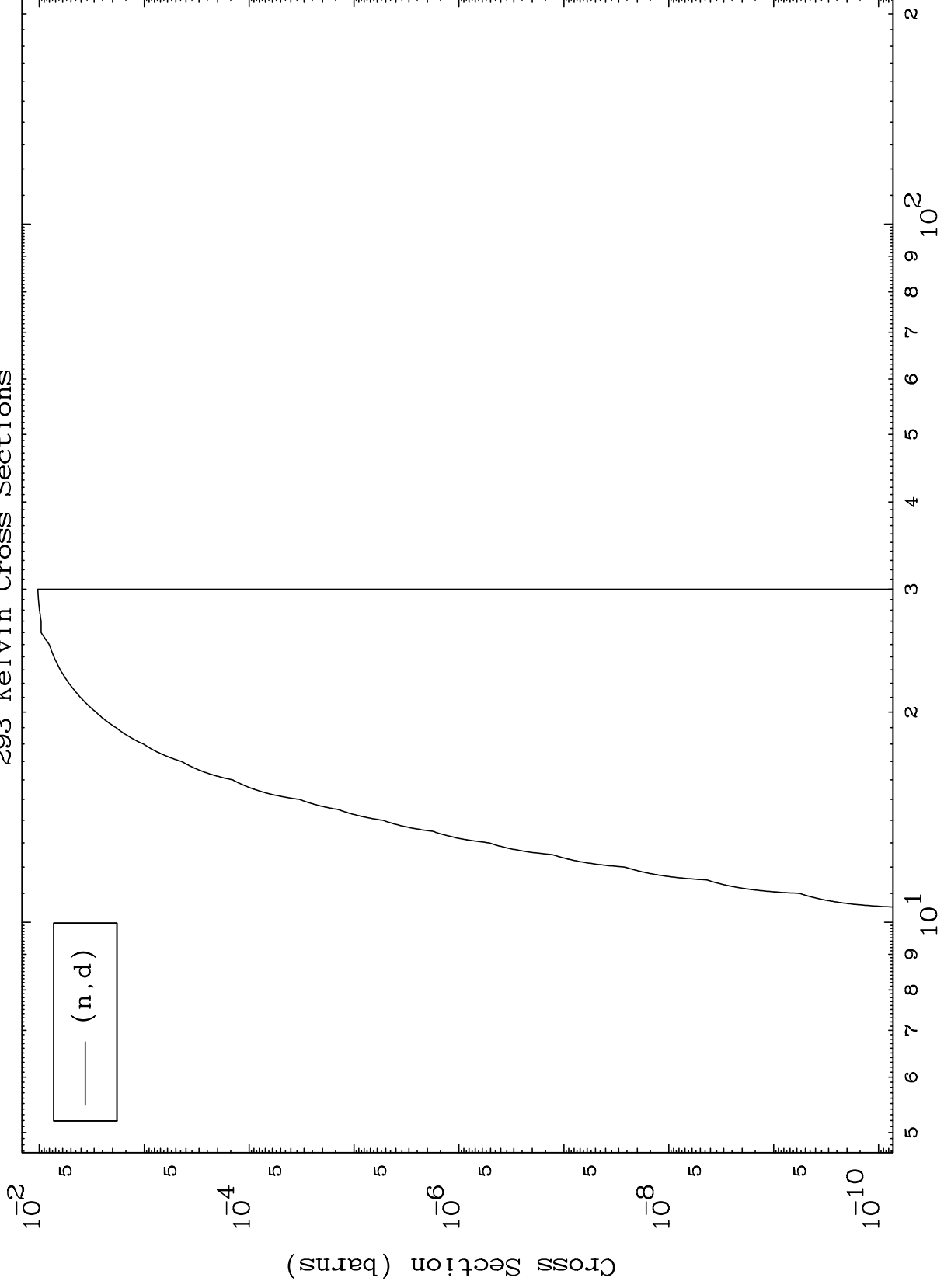
Incident Energy (MeV)

12

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(n,d) Levels  
293 Kelvin Cross Sections

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(n,d)

13

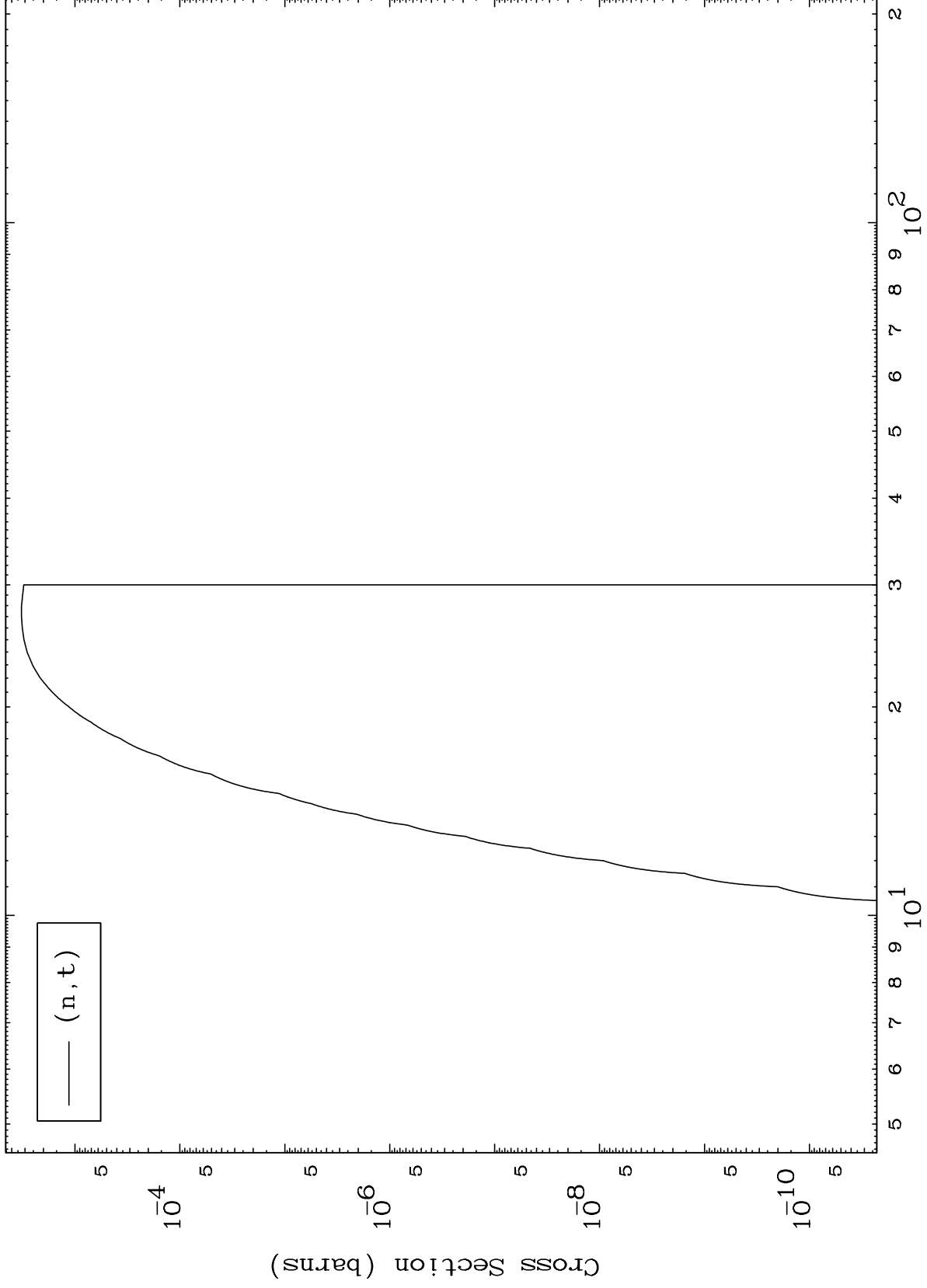
Incident Energy (MeV)

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(n,t) Levels  
293 Kelvin Cross Sections

92-U -236



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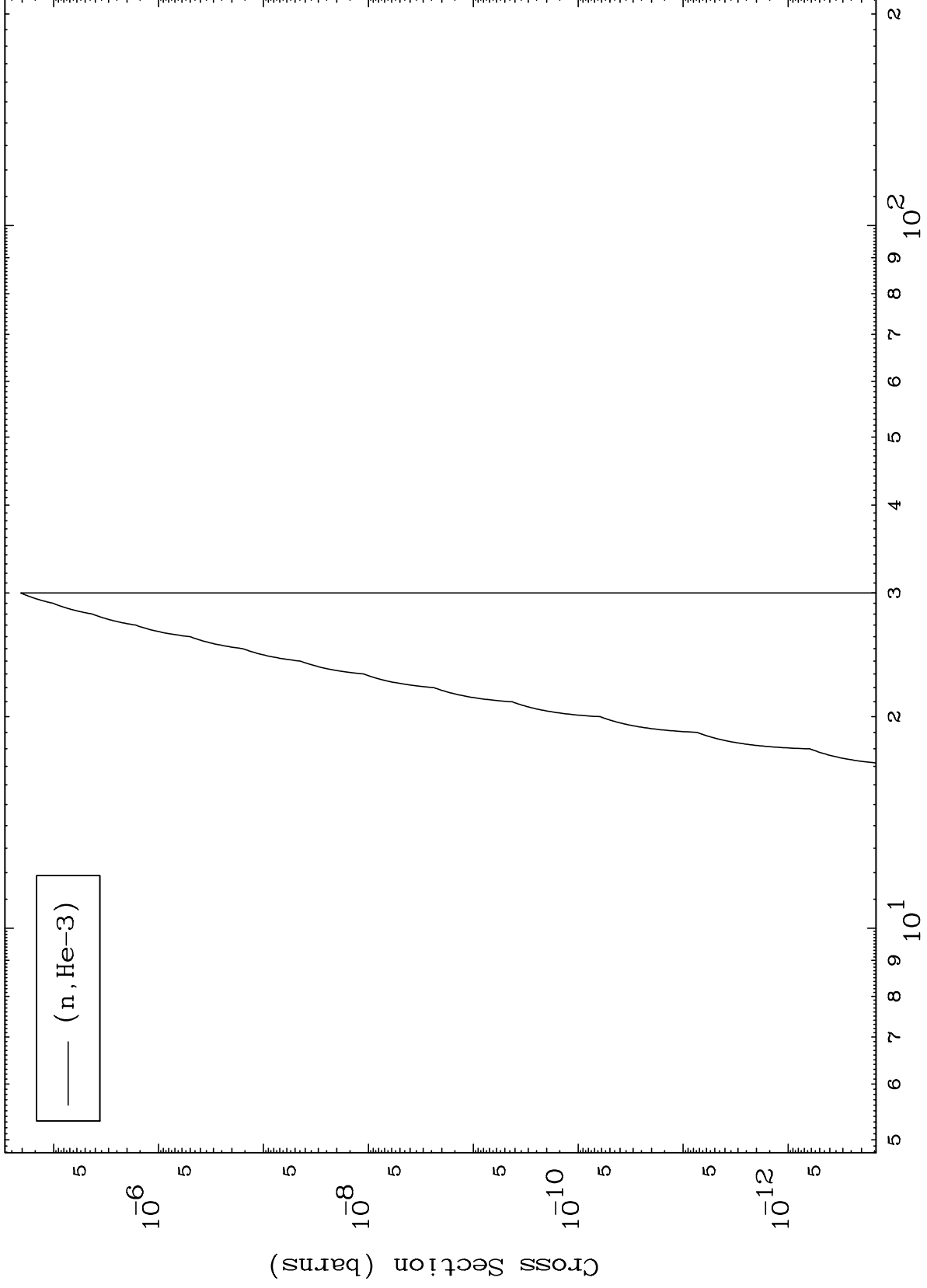
Incident Energy (MeV)

92-U -236

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(n,He3) Levels  
293 Kelvin Cross Sections

92-U -236



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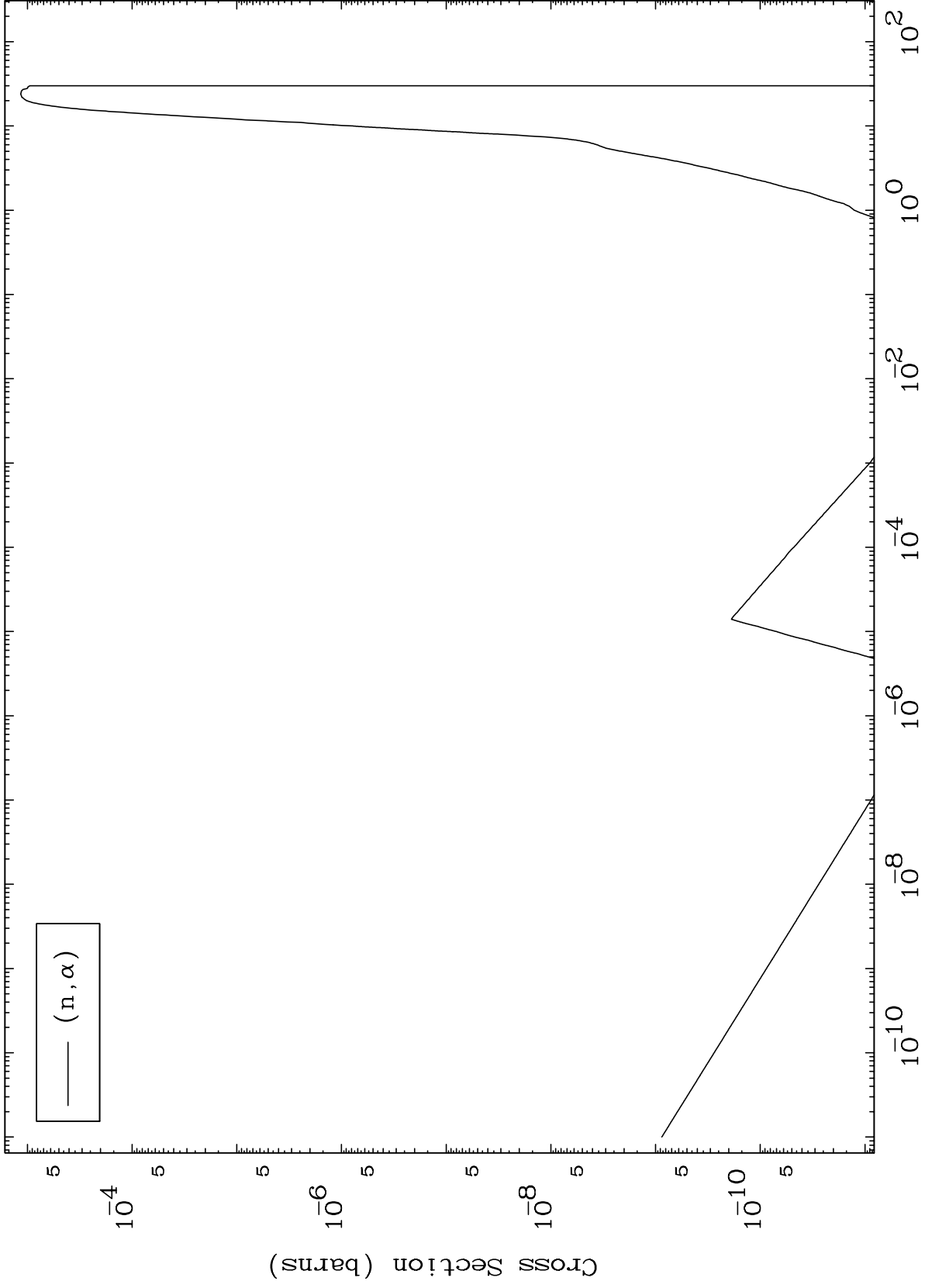
Incident Energy (MeV)

92-U -236

MAT 9231

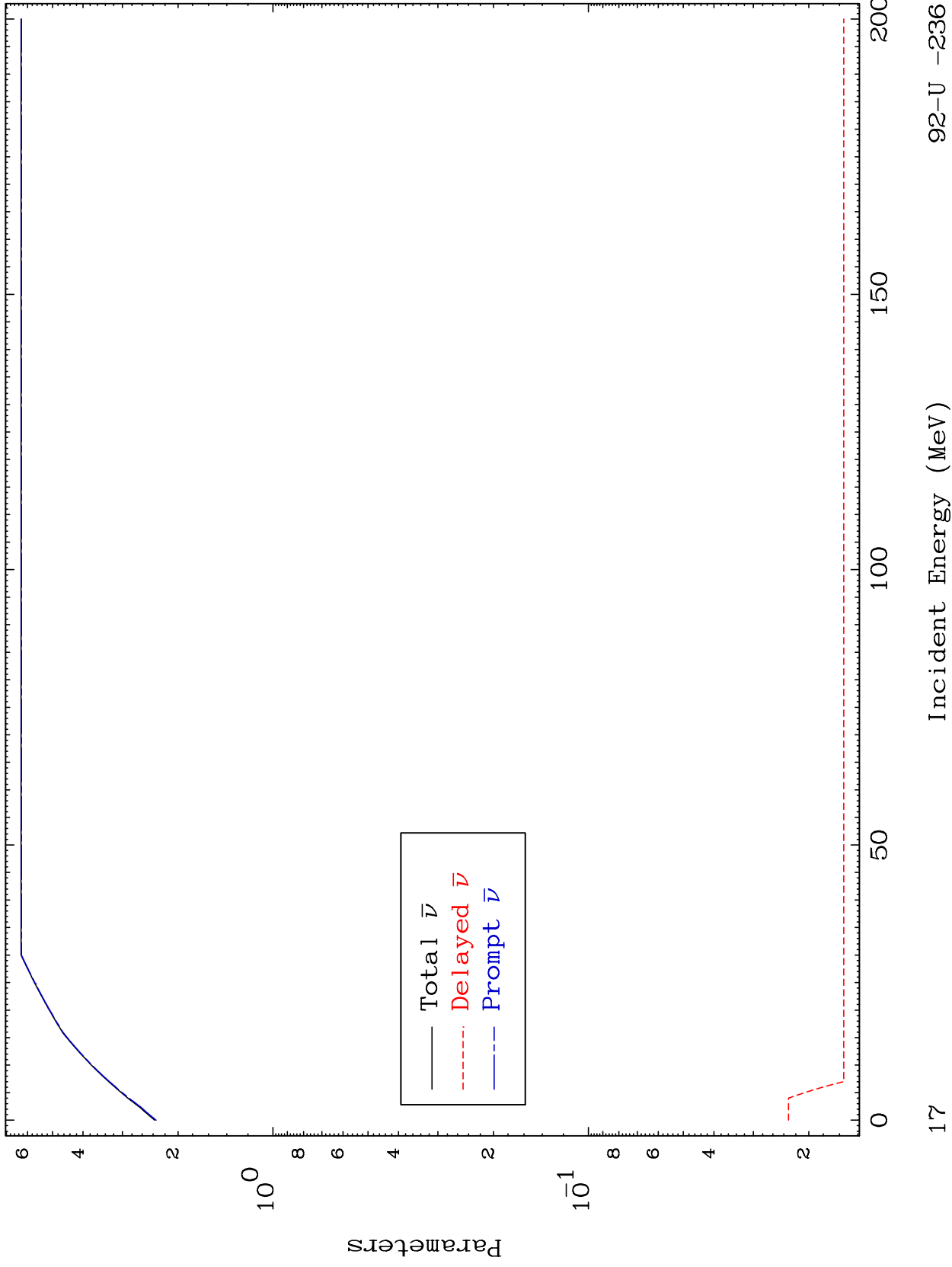
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

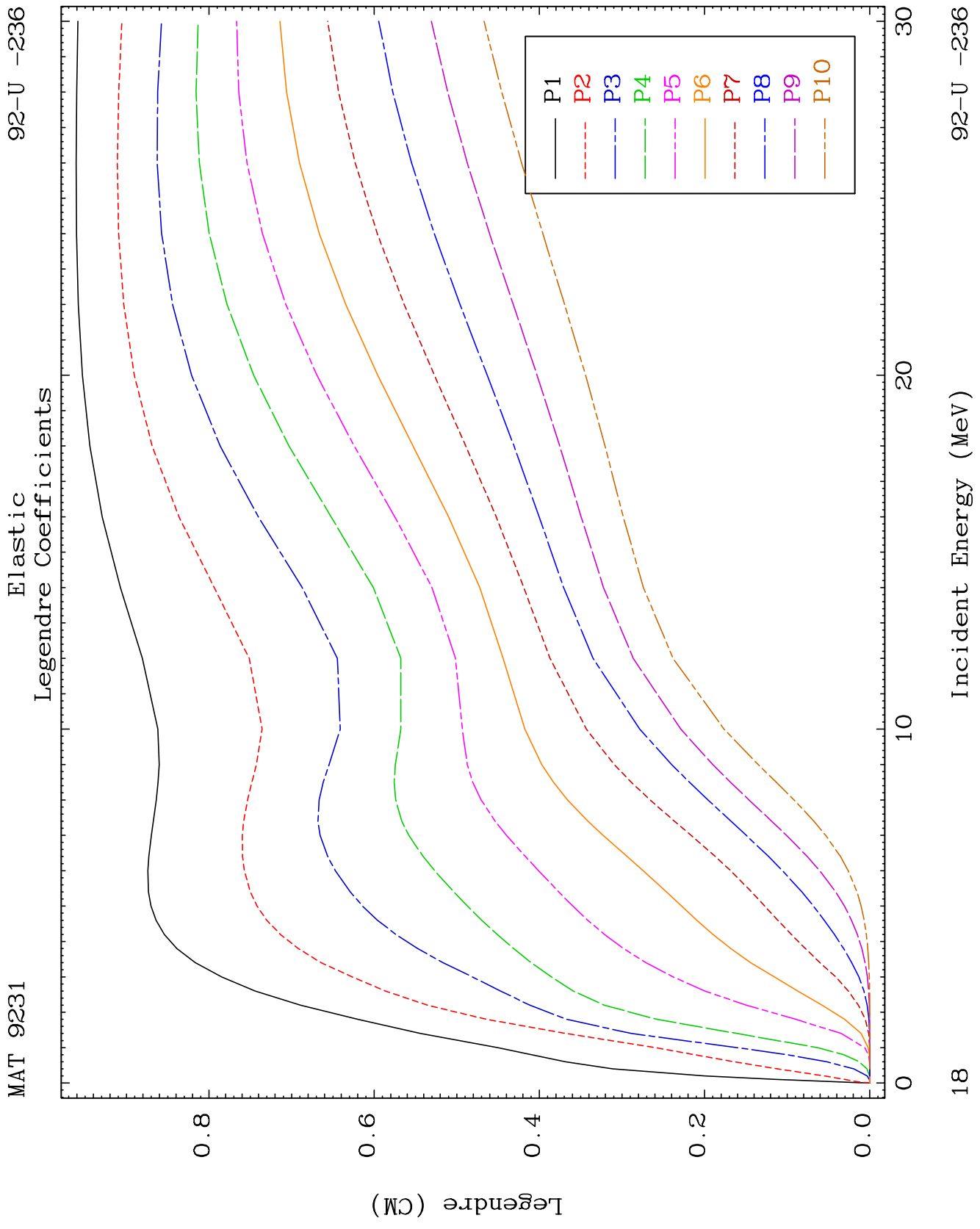
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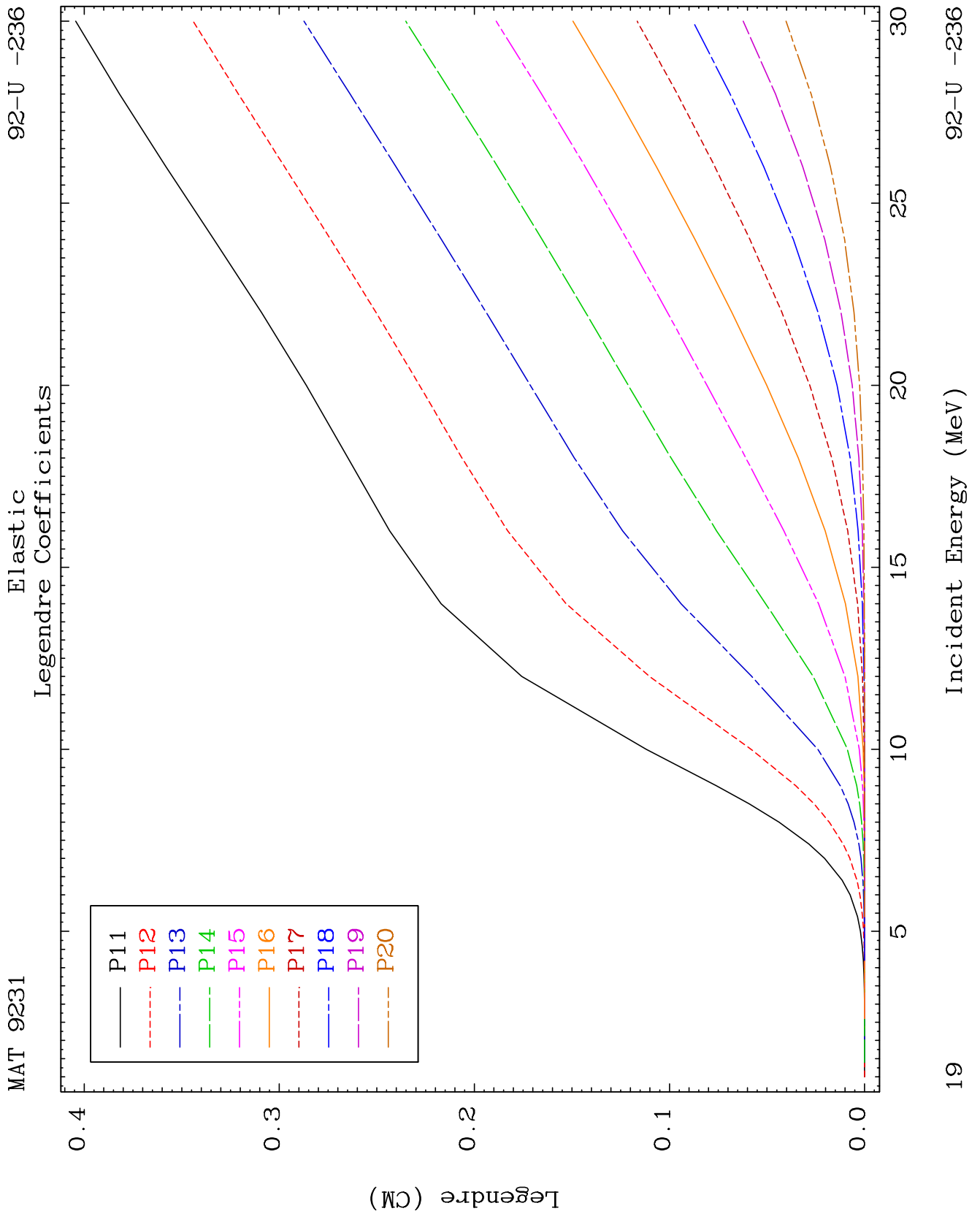


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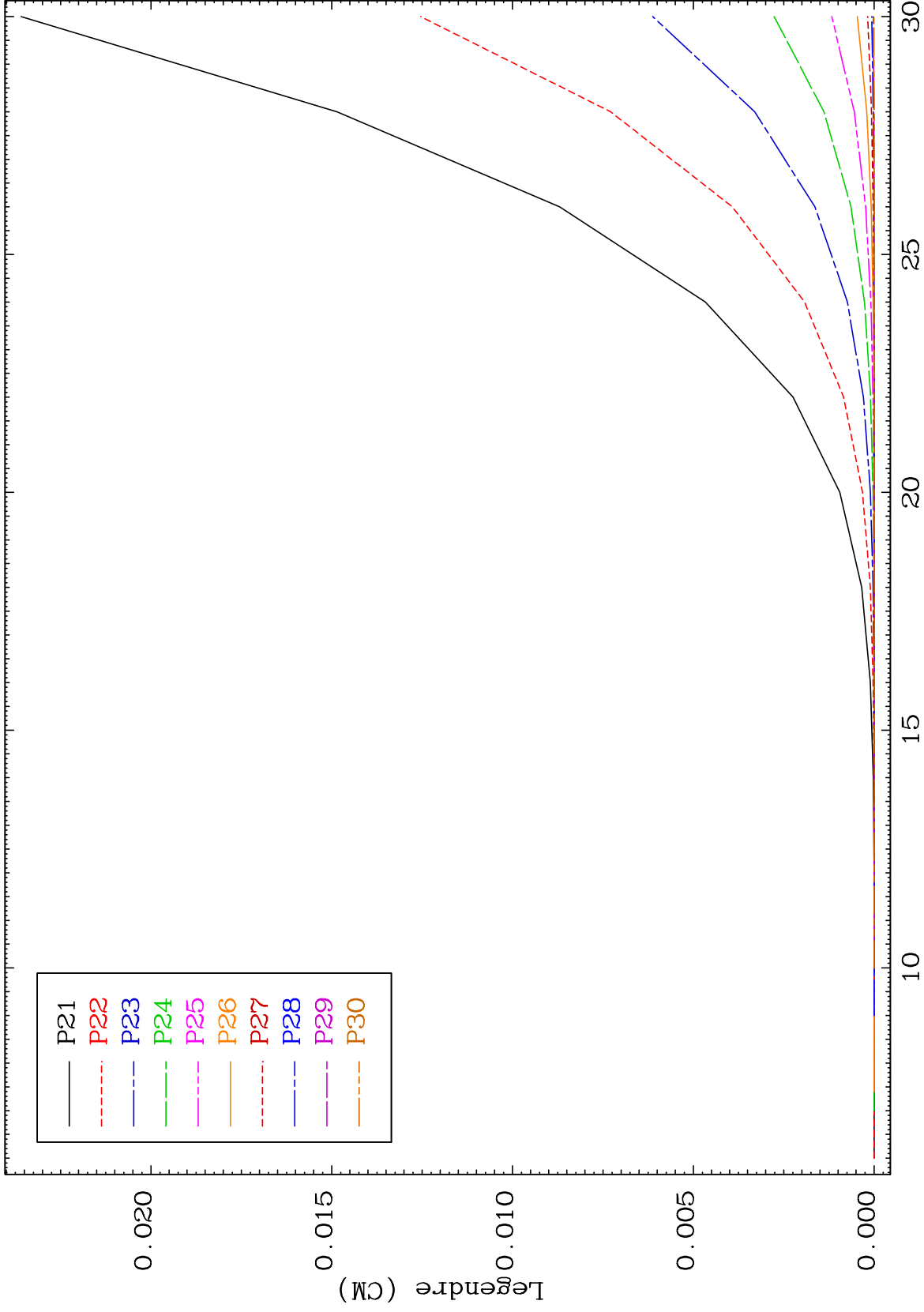




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Elastic Legendre Coefficients

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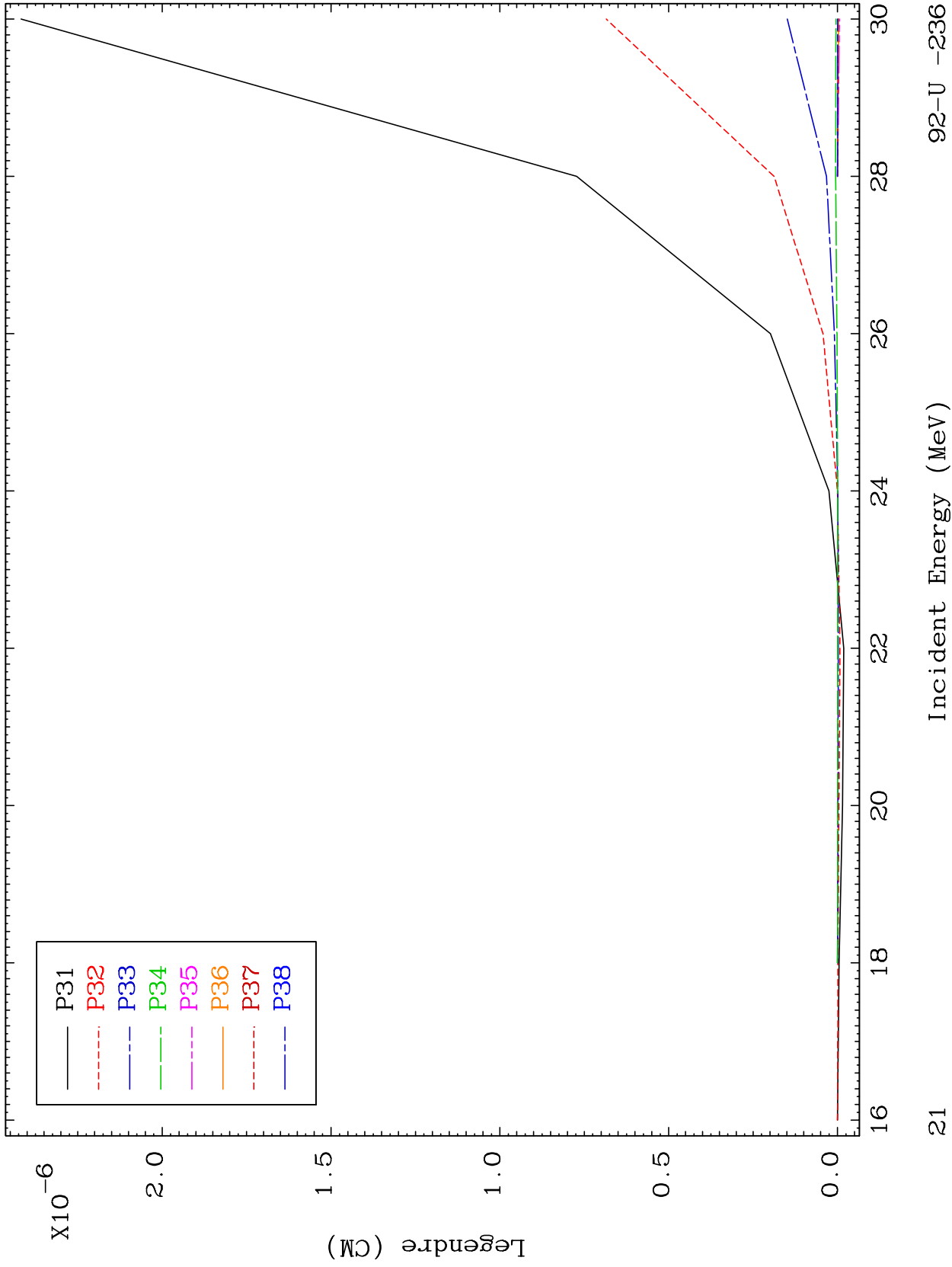
Incident Energy (MeV)

92-U -236

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Elastic  
Legendre Coefficients

92-U -236



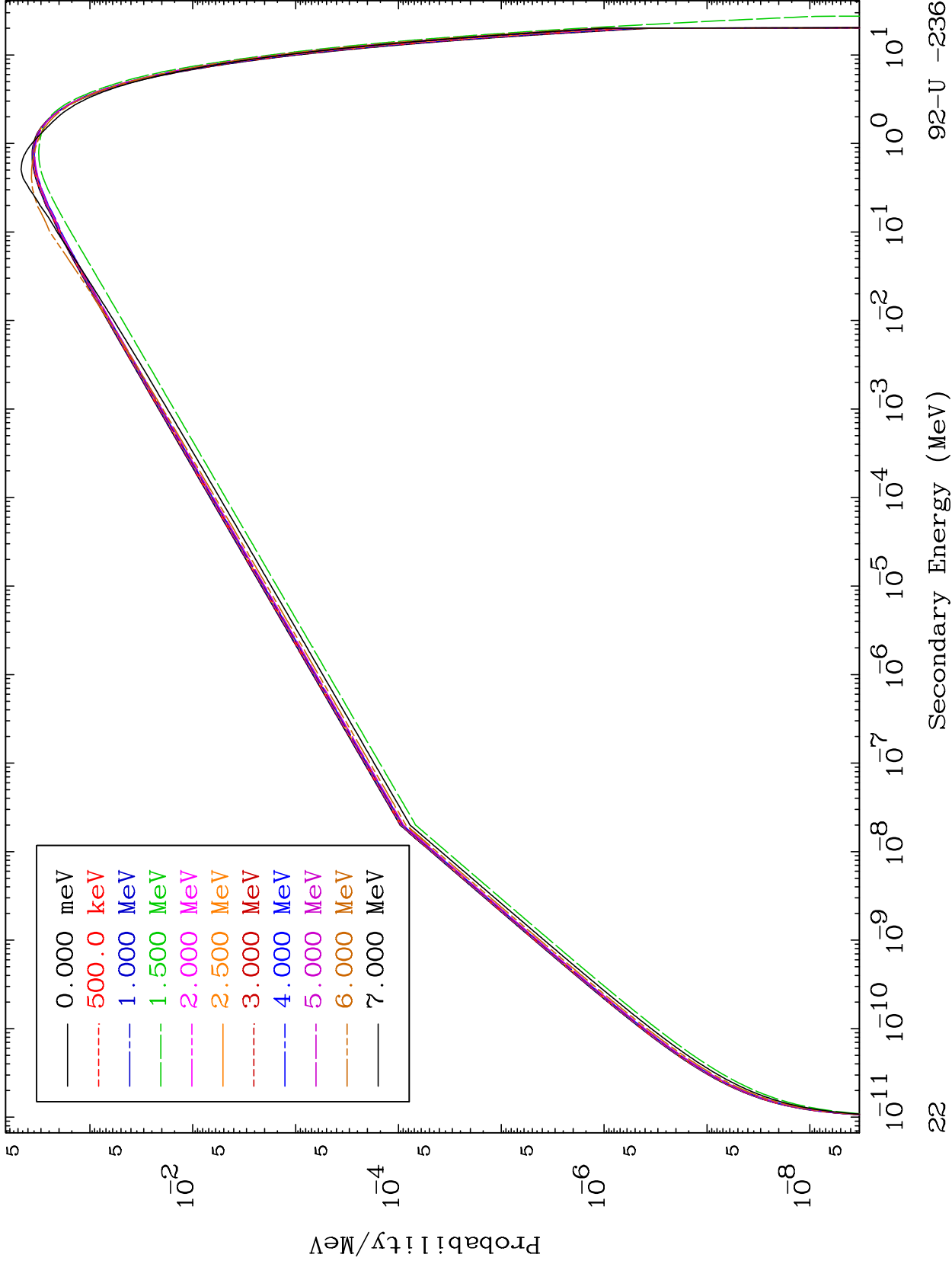
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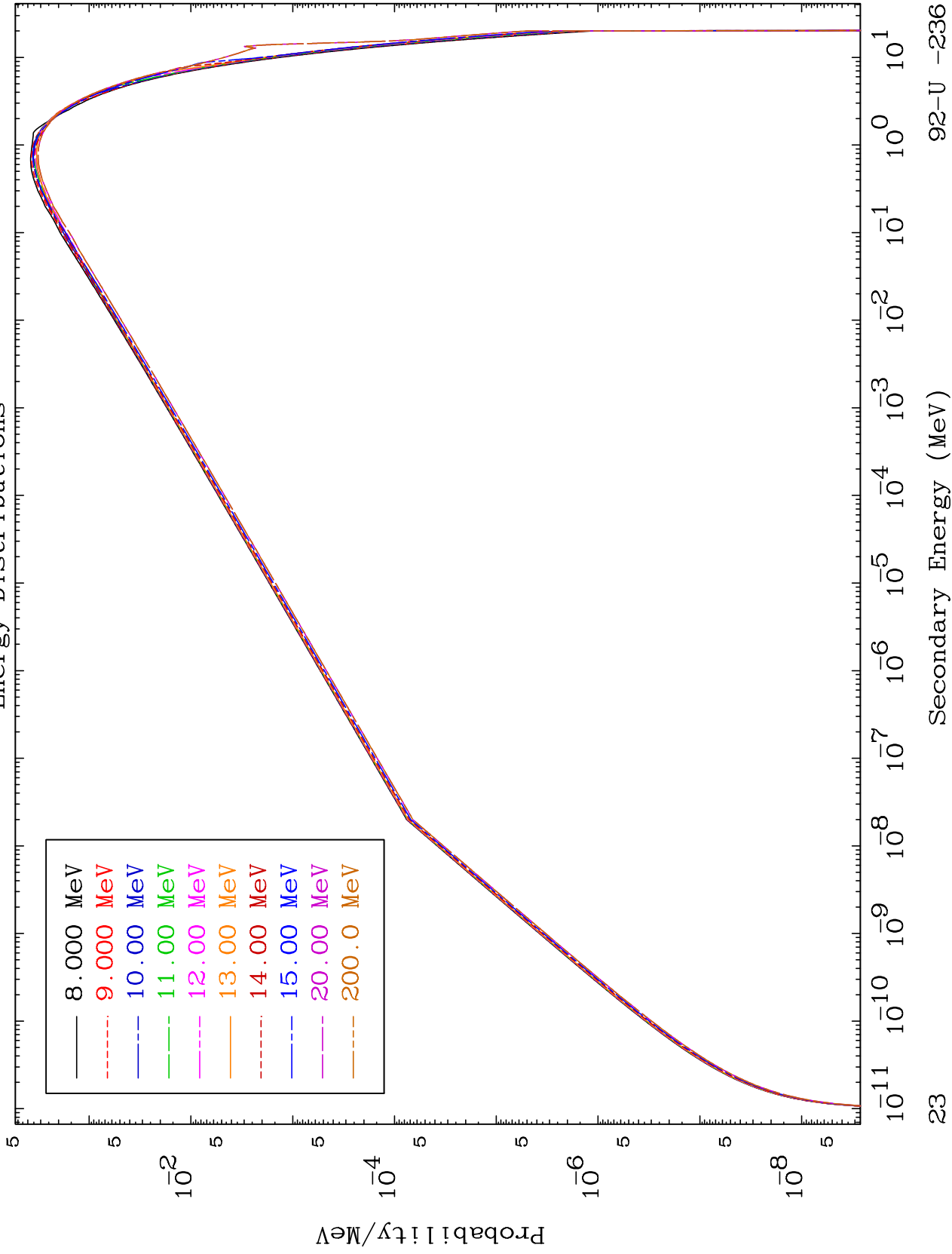
### Fission Energy Distributions

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Fission Energy Distributions

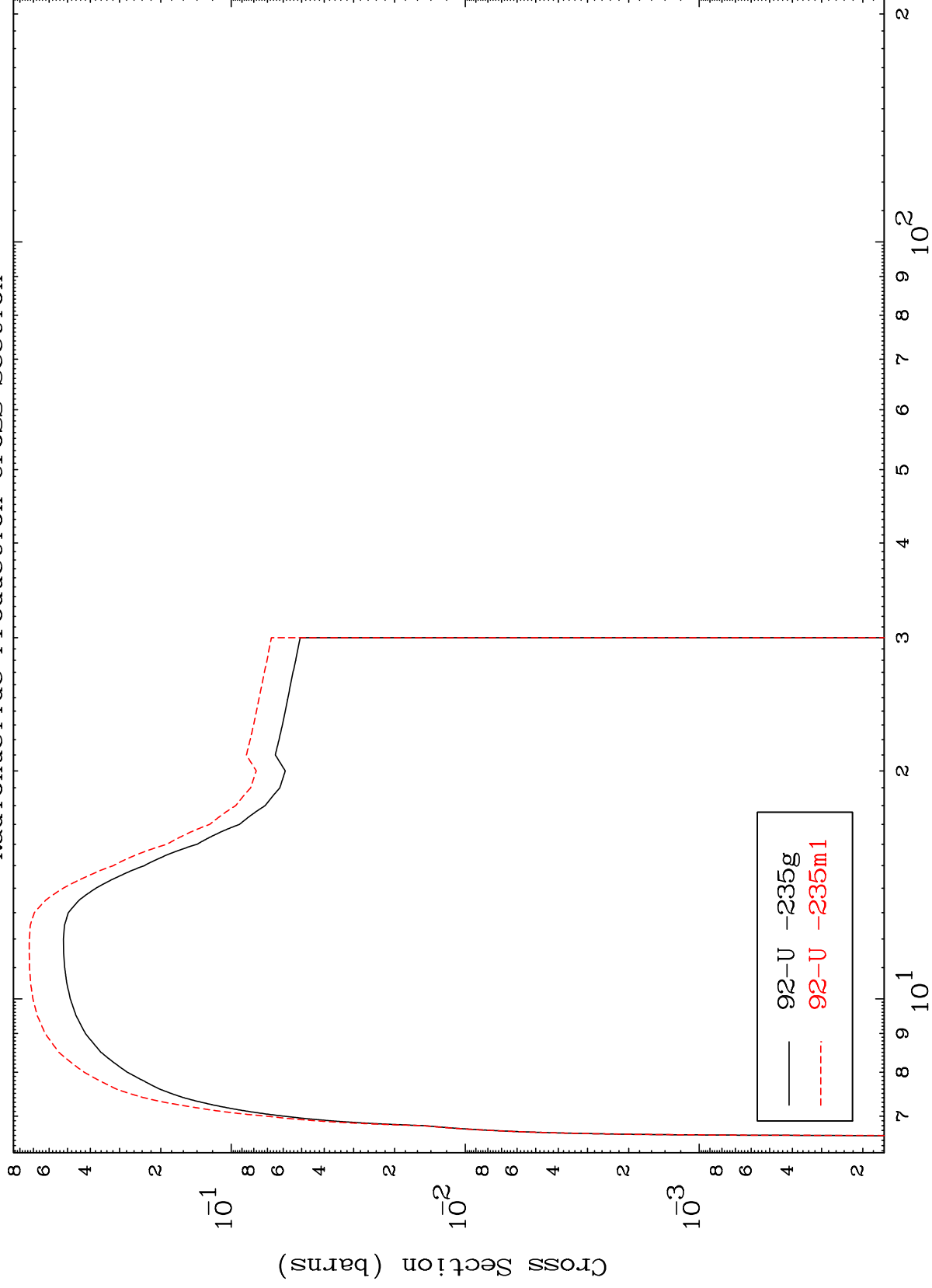


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(n,2n)

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Radionuclide Production Cross Section

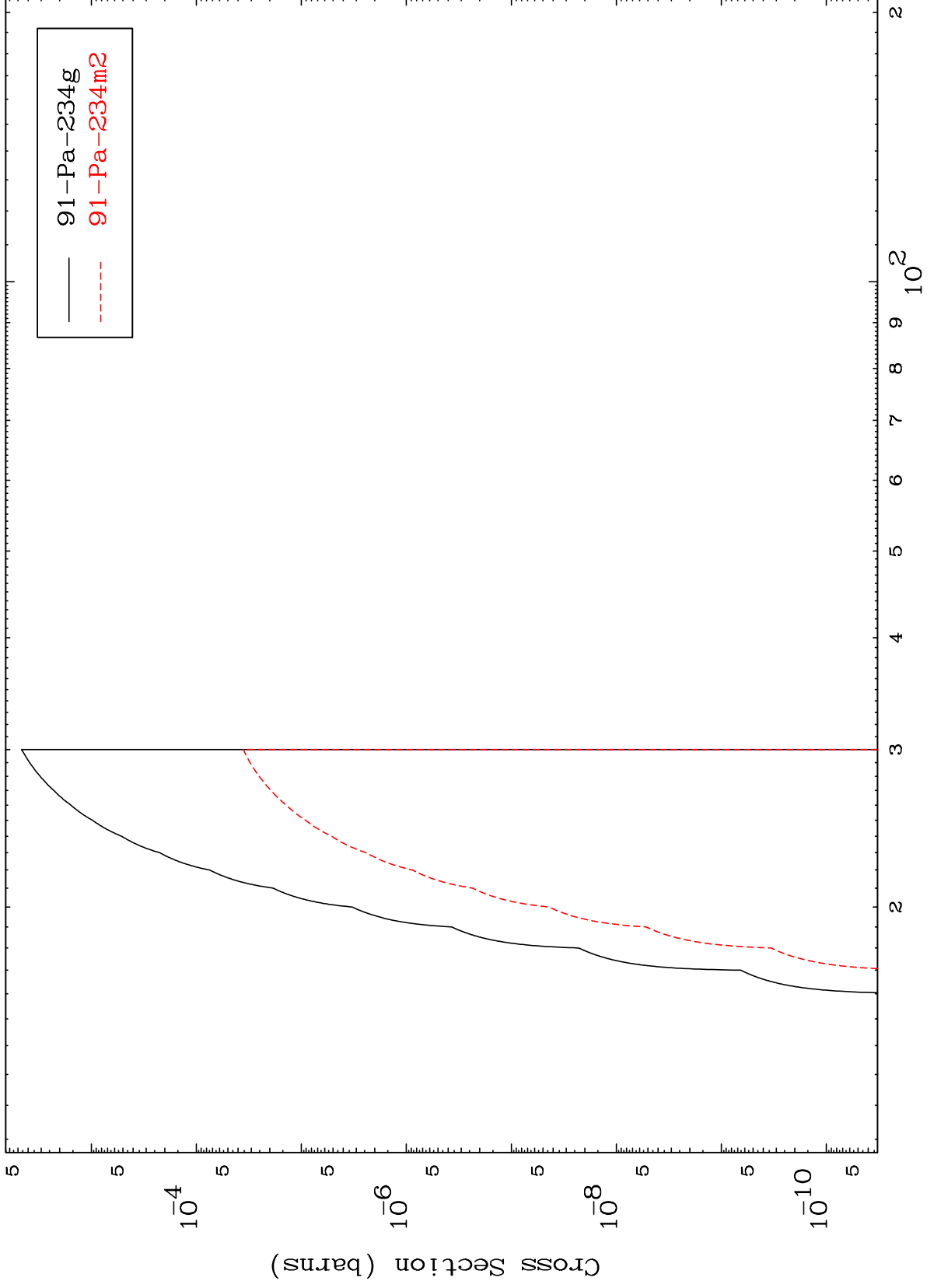


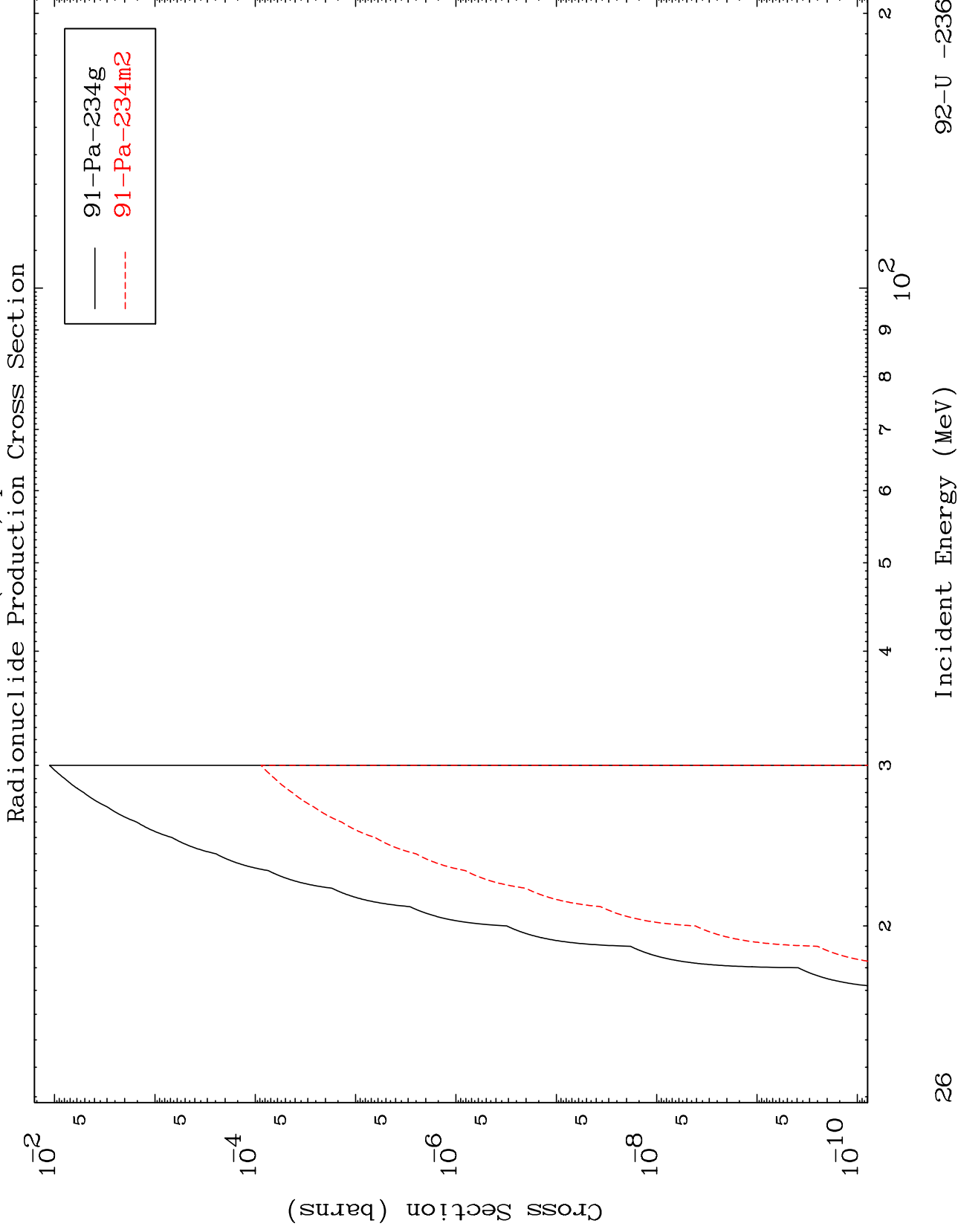
24

Incident Energy (MeV)

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Radionuclide Production Cross Section

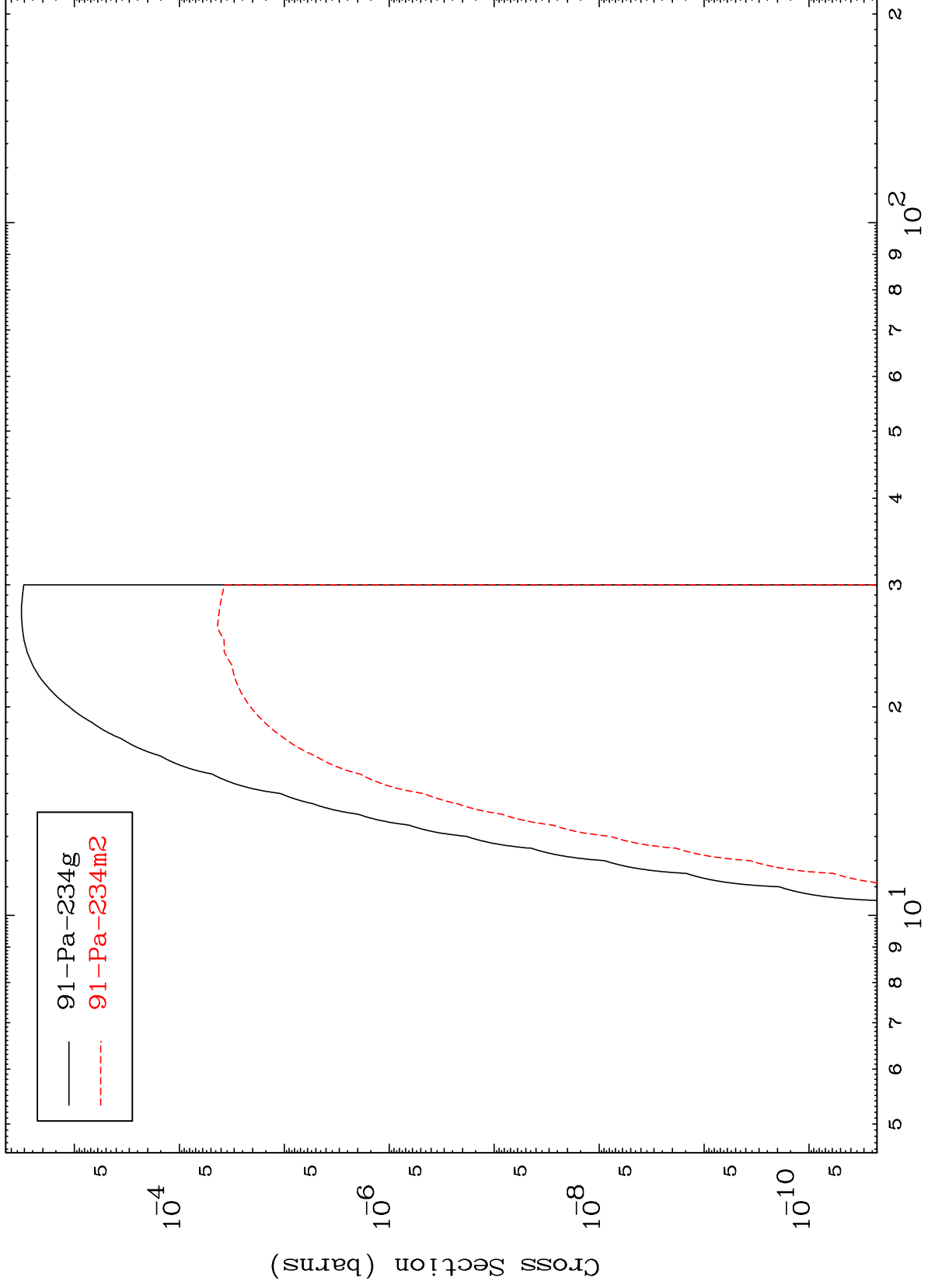




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Radionuclide Production Cross Section (n,t)



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Incident Energy (MeV)

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