

Program EVALPLOT  
(Version 2021-1)

by

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(Present Contact Information)

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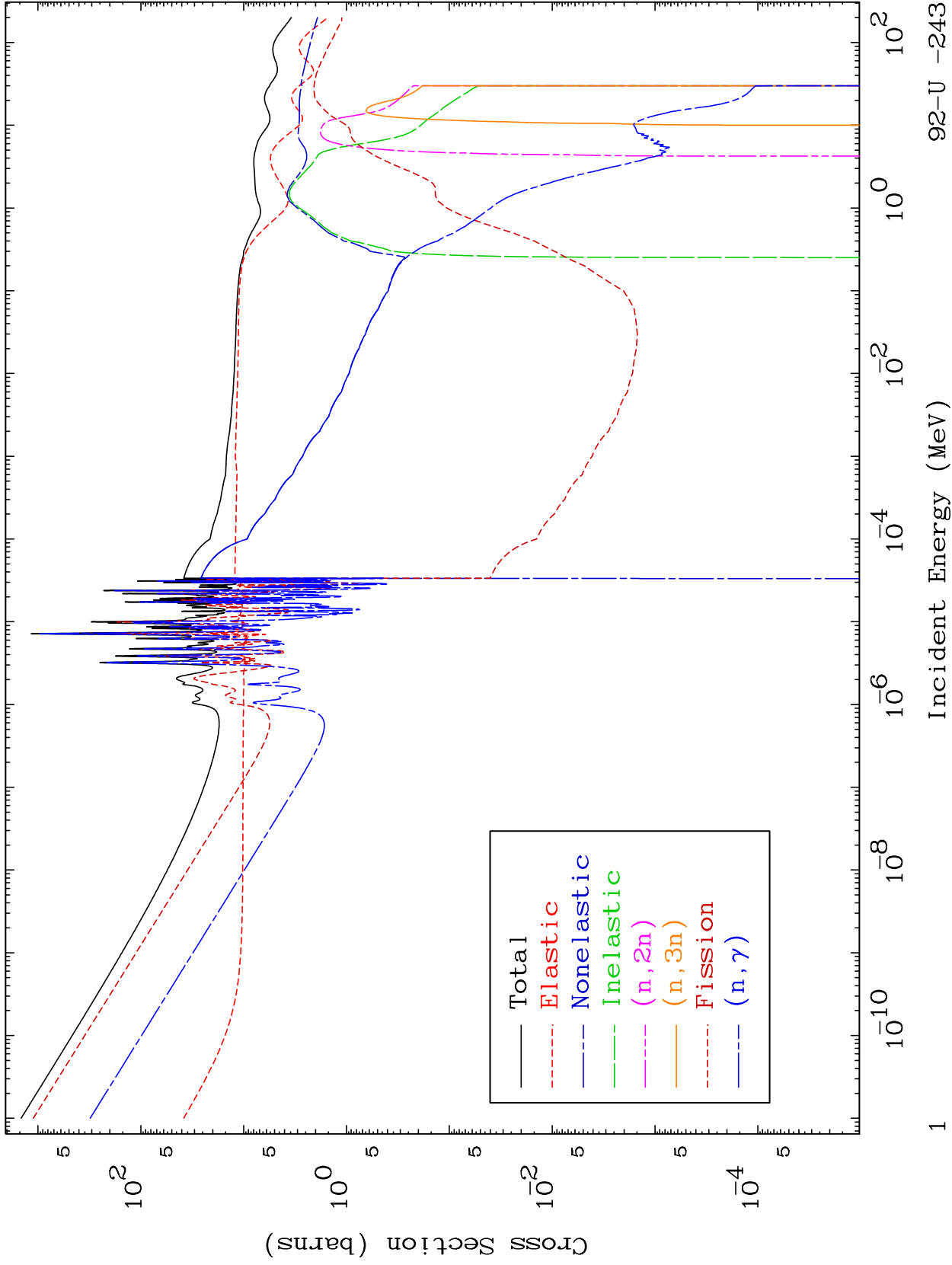
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9252

Neutron Major  
293 Kelvin Cross Sections

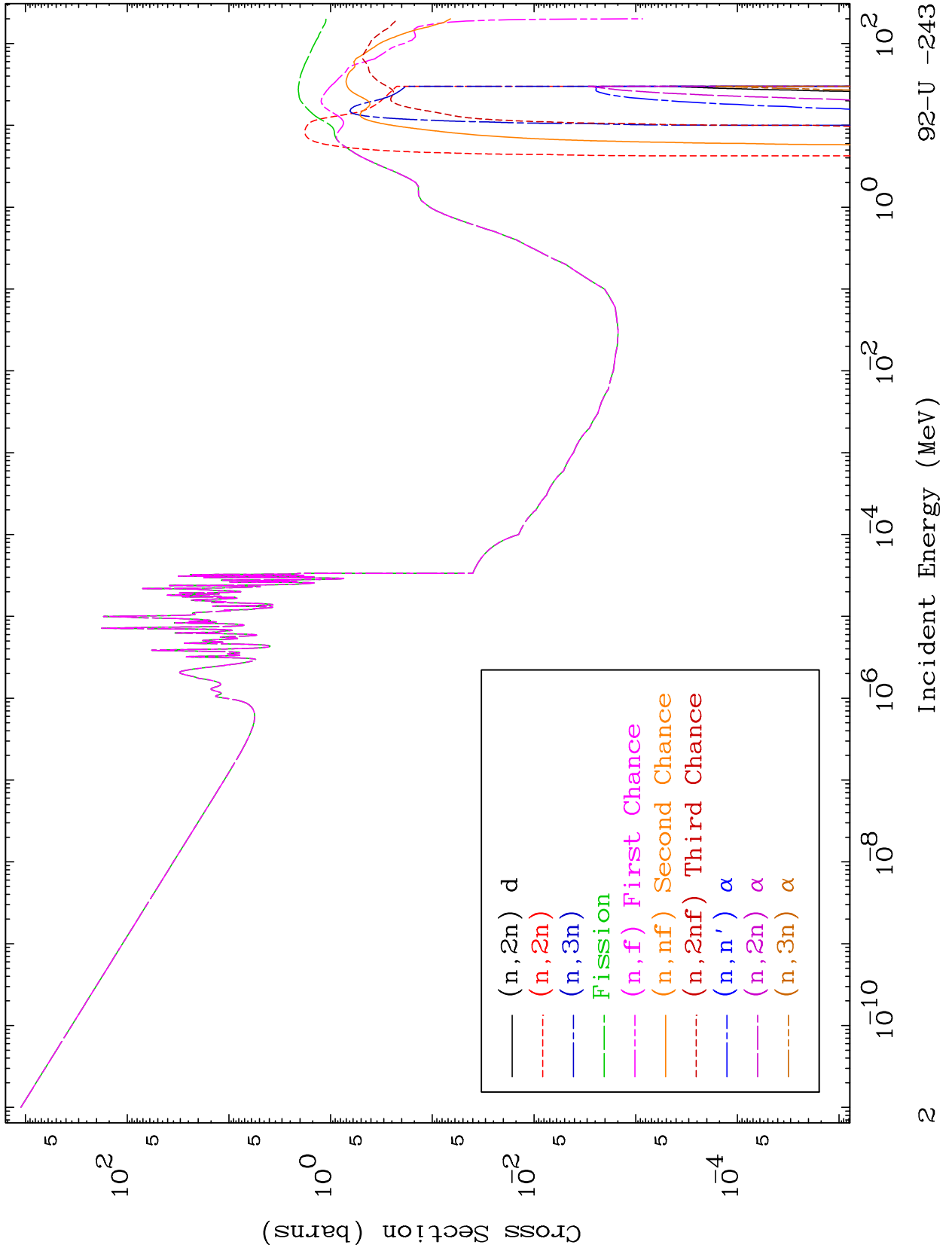
92-U -243



MAT 9252

Neutron Absorption  
293 Kelvin Cross Sections

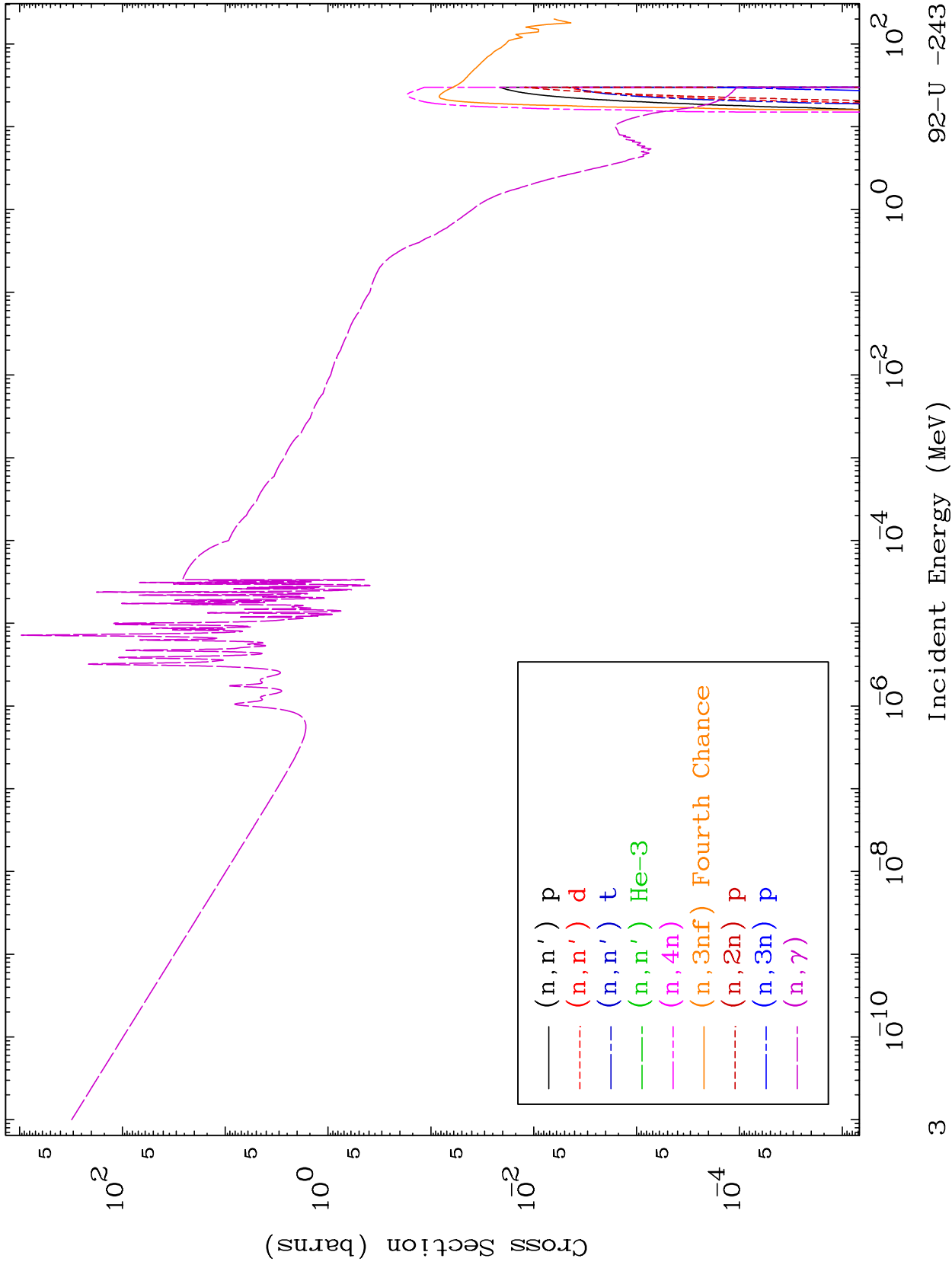
92-U -243

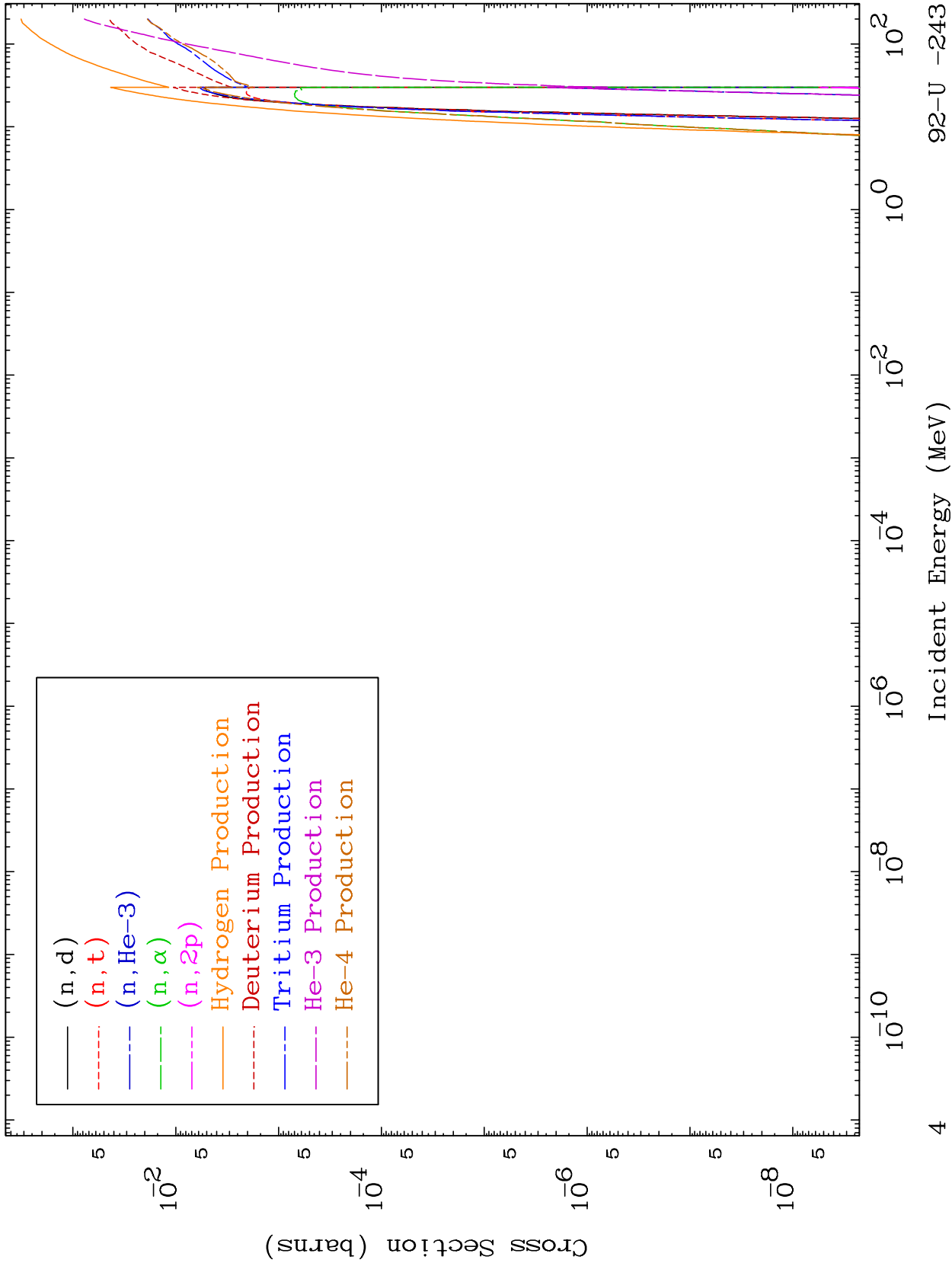


MAT 9252

Neutron Absorption  
293 Kelvin Cross Sections

92-U -243

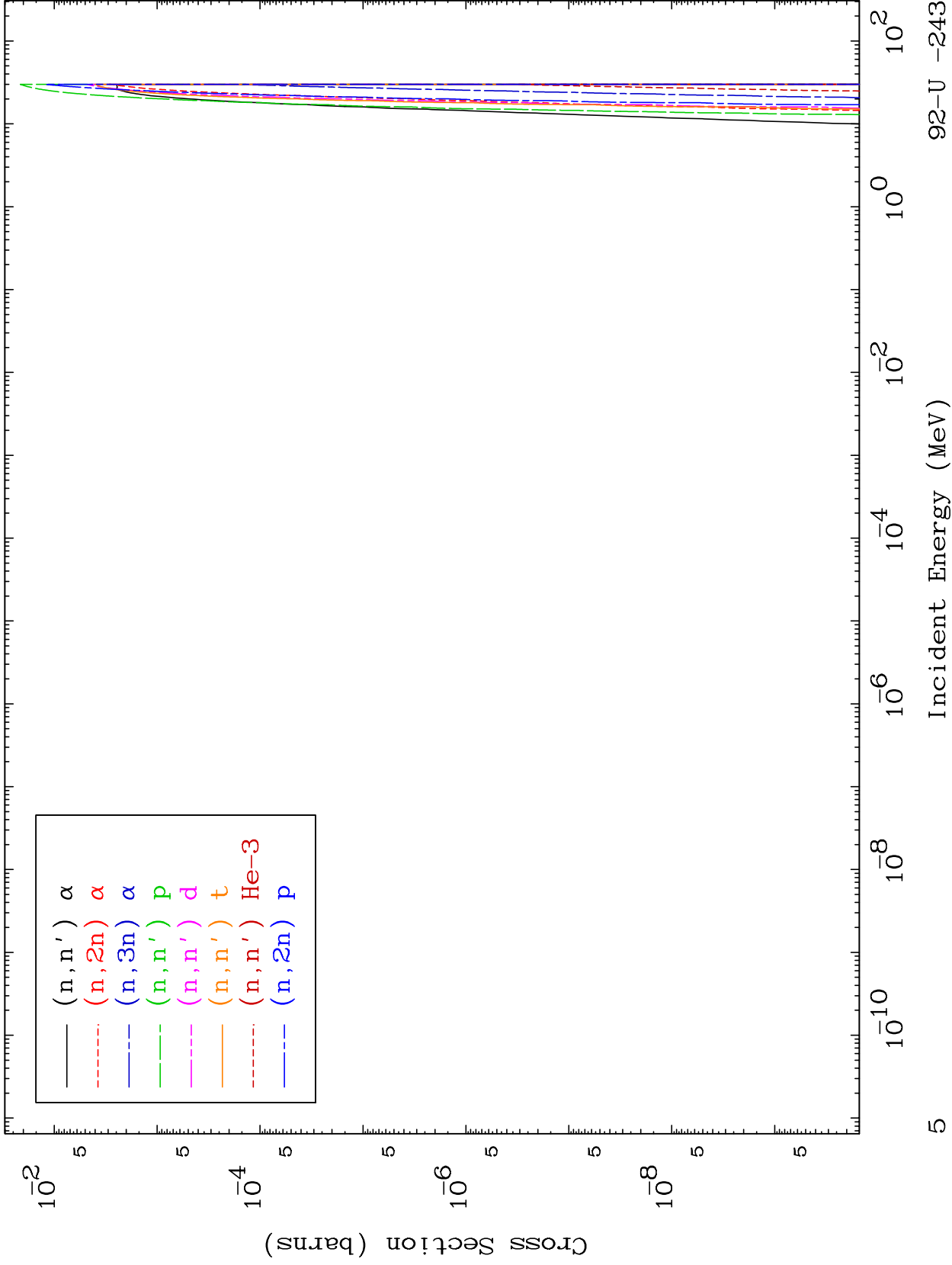




MAT 9252

Charged Particle  
293 Kelvin Cross Sections

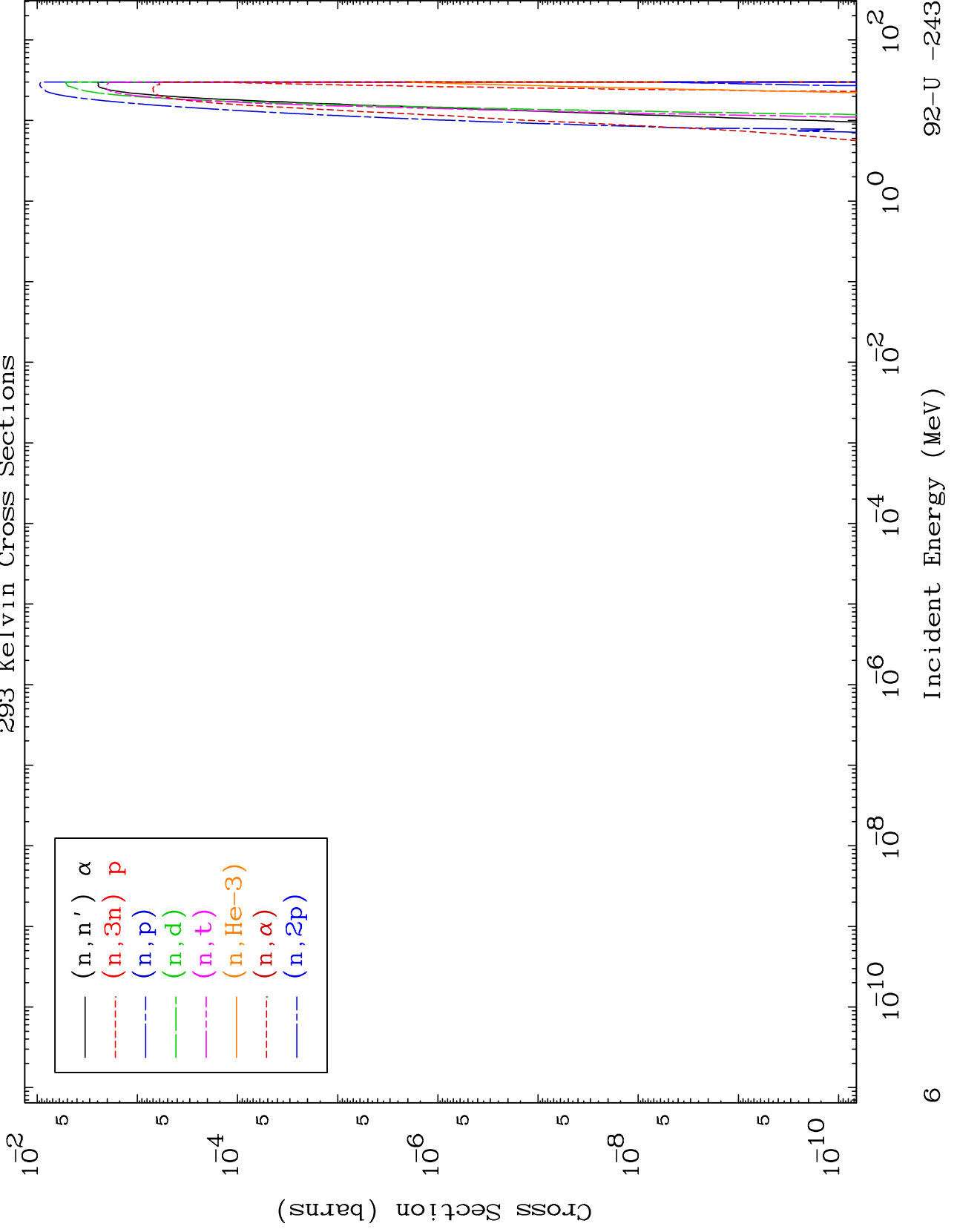
92-U -243



MAT 9252

Charged Particle  
293 Kelvin Cross Sections

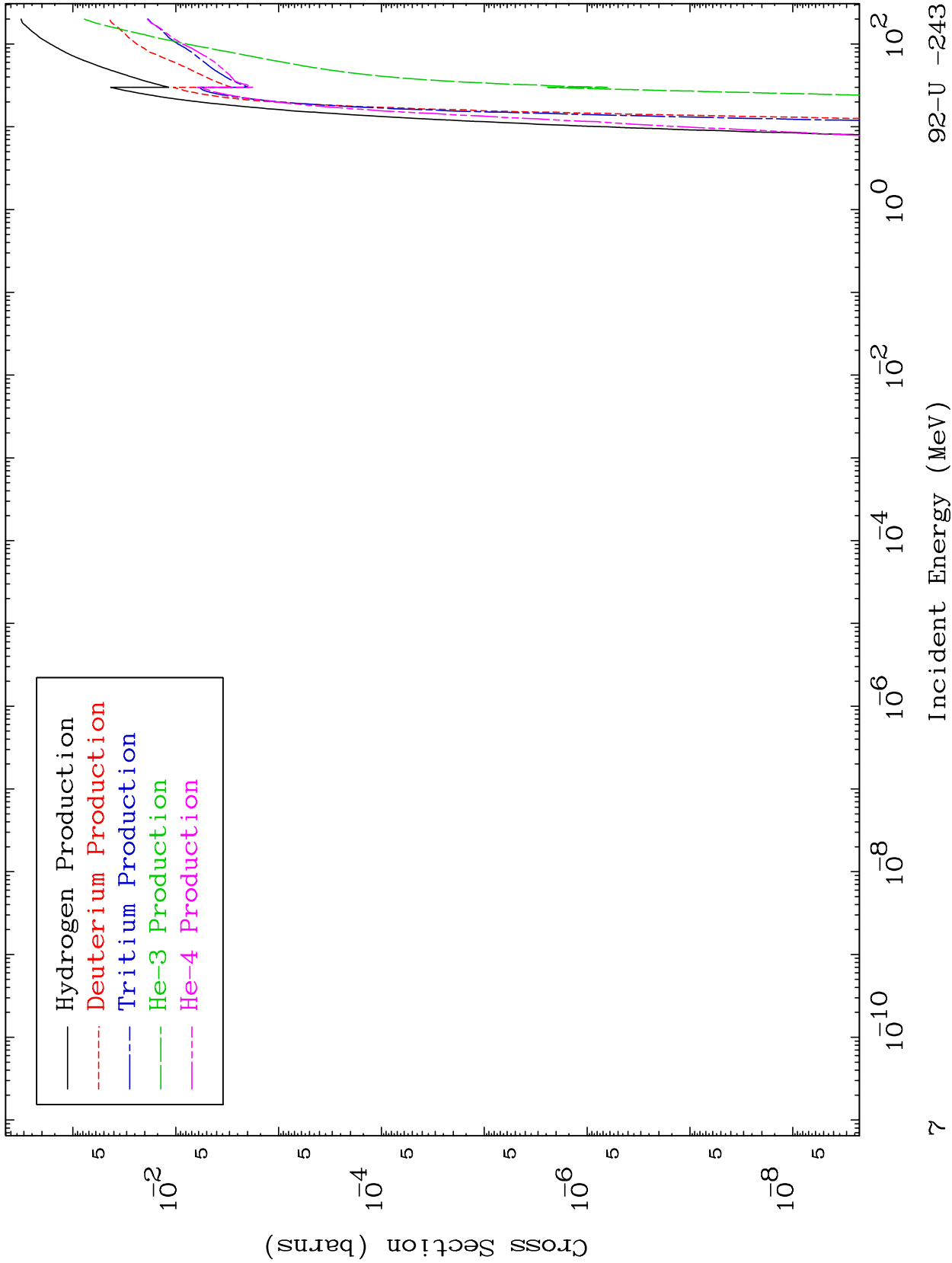
92-U -243



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Particle Production  
293 Kelvin Cross Sections

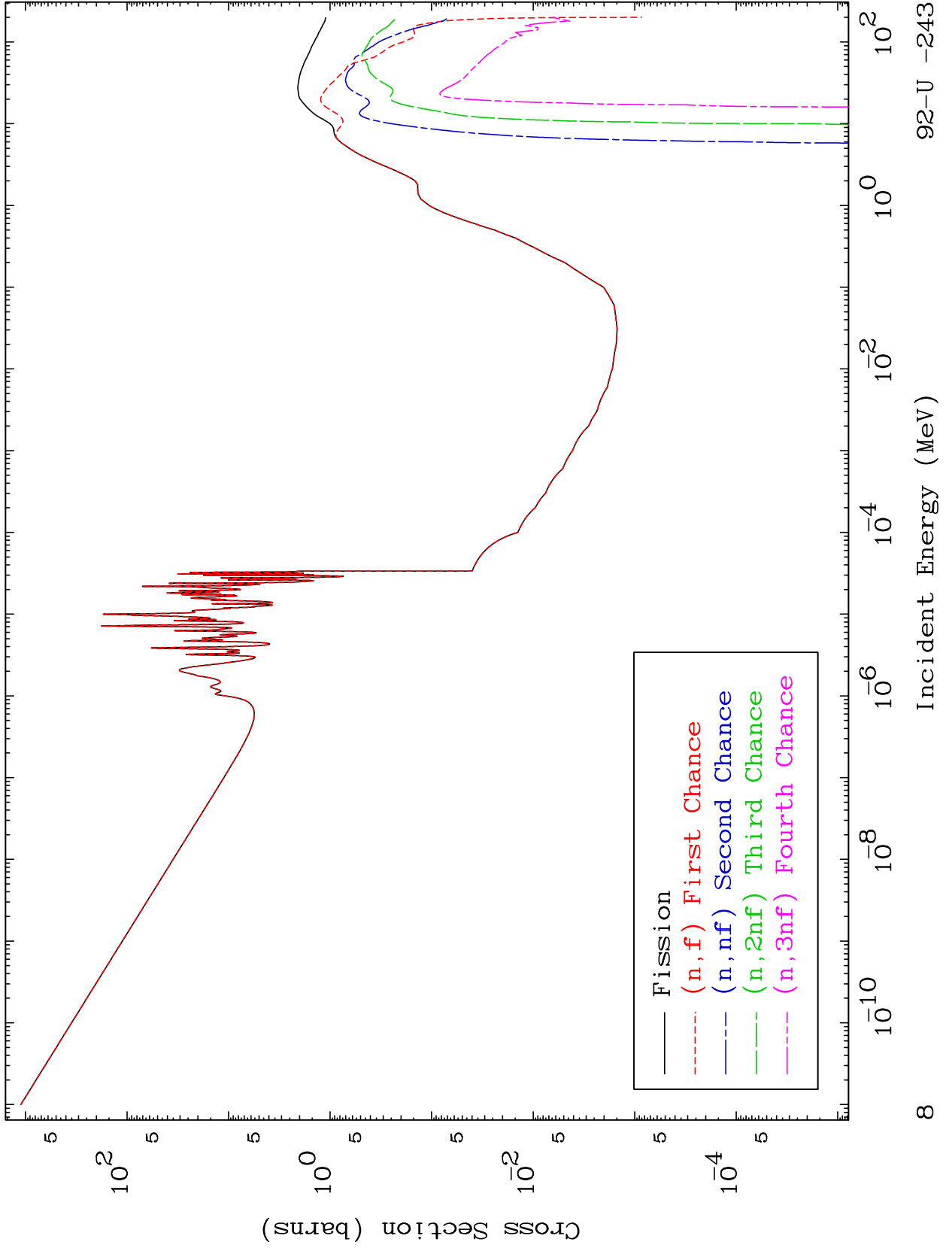
92-U -243



MAT 9252

Fission  
293 Kelvin Cross Sections

92-U -243

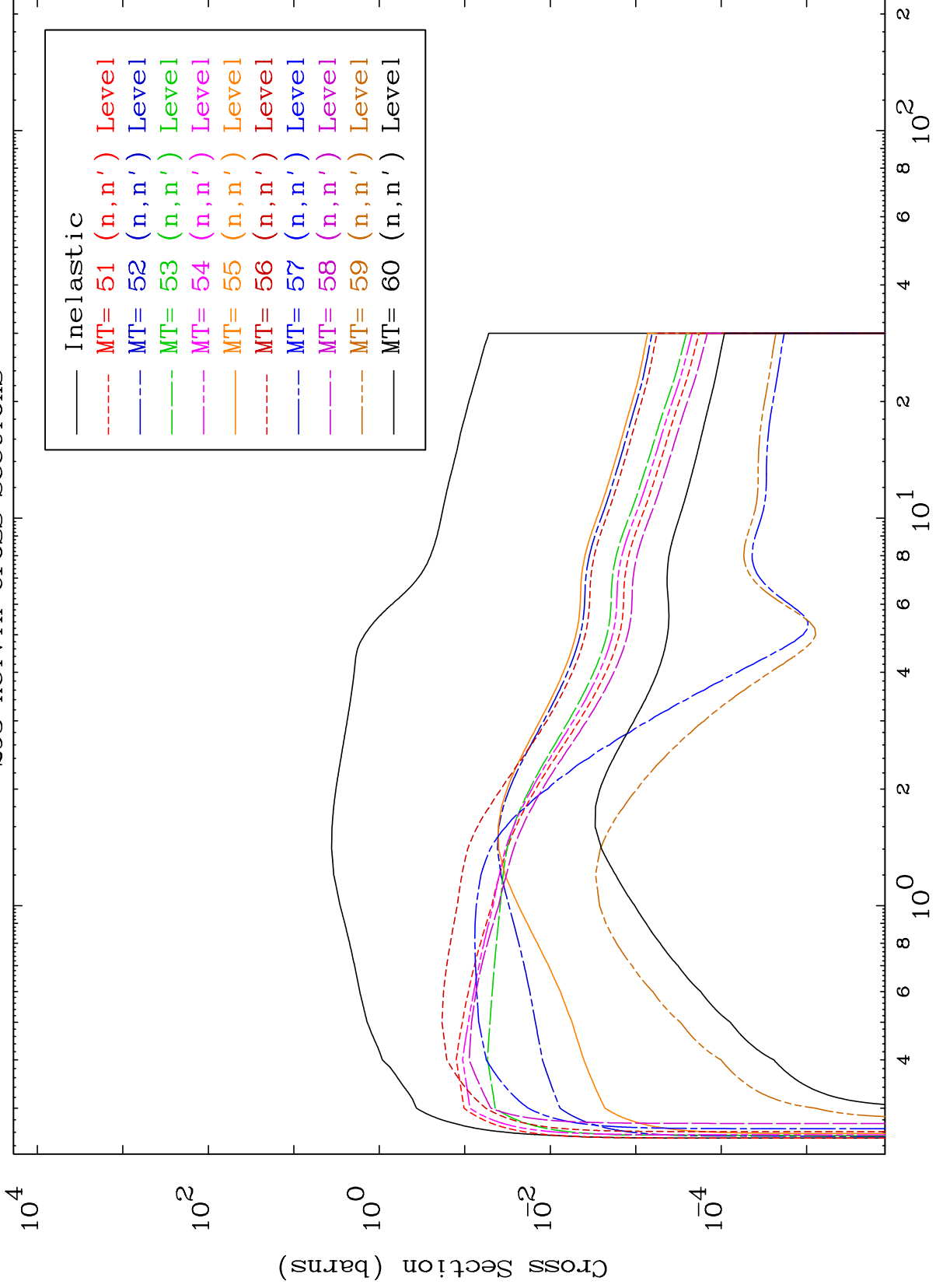


MAT 9252

(n,n') Levels

92-U -243

293 Kelvin Cross Sections

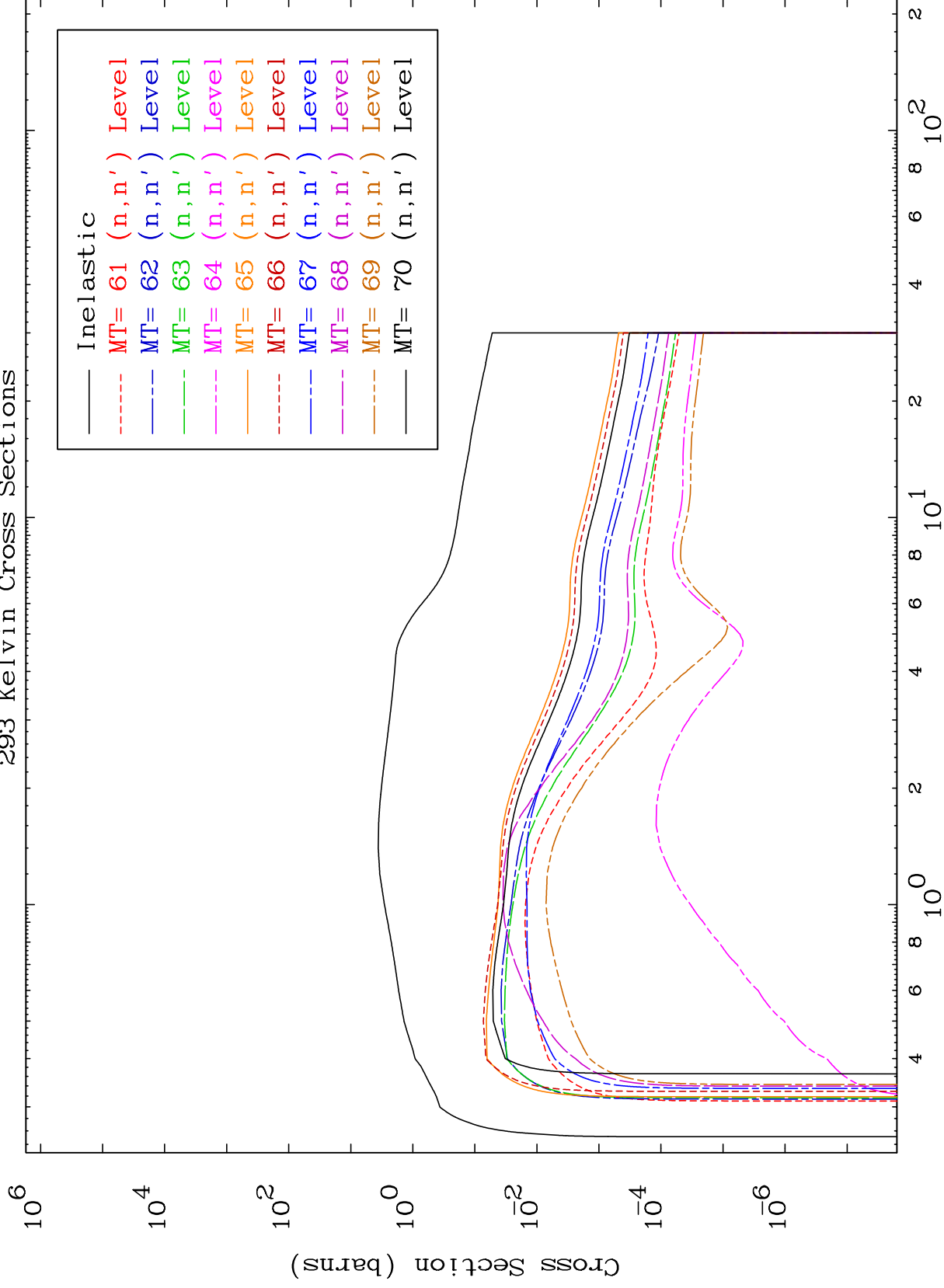


MAT 9252

(n,n') Levels

92-U -243

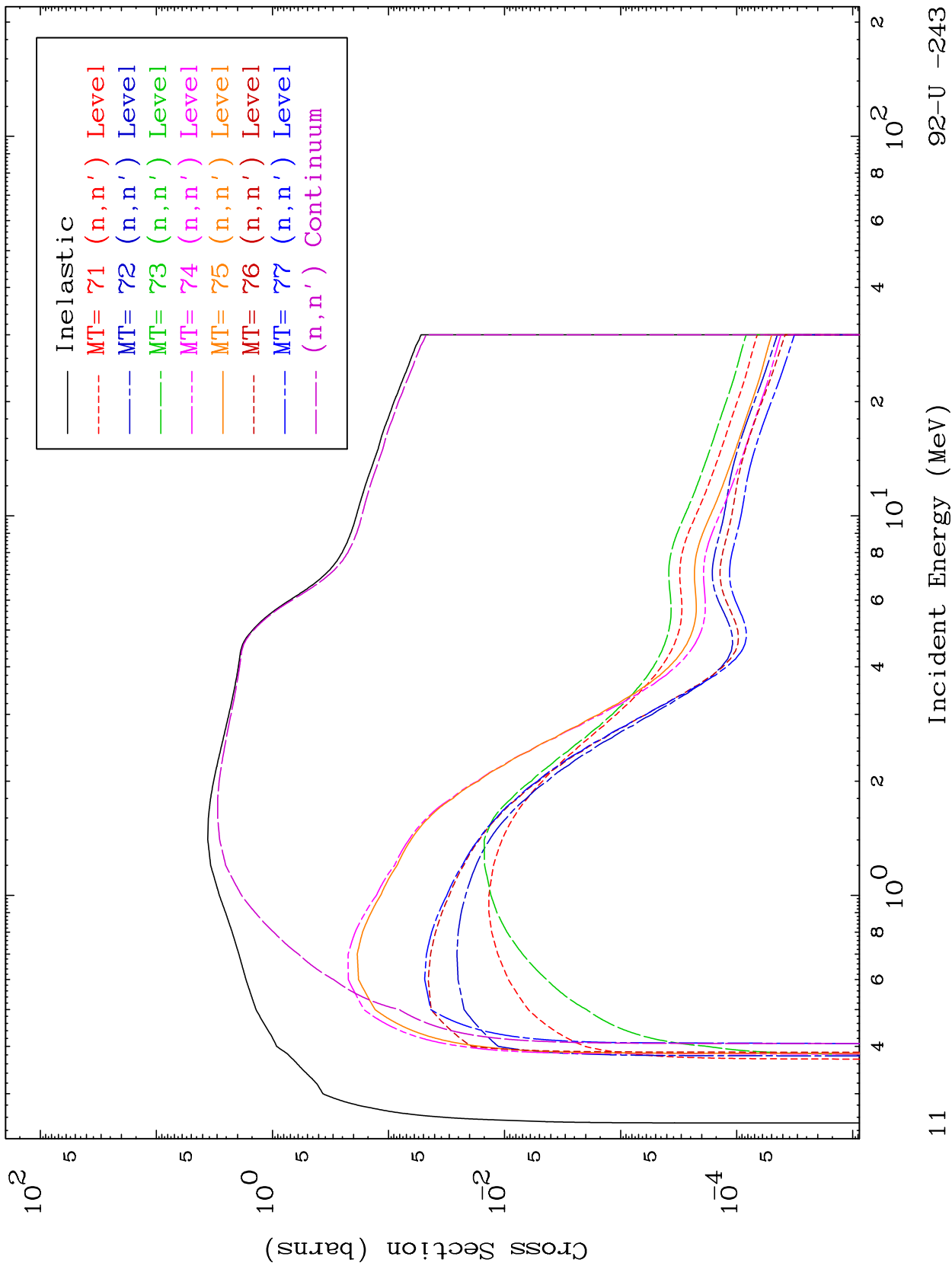
293 Kelvin Cross Sections



Incident Energy (MeV)

10

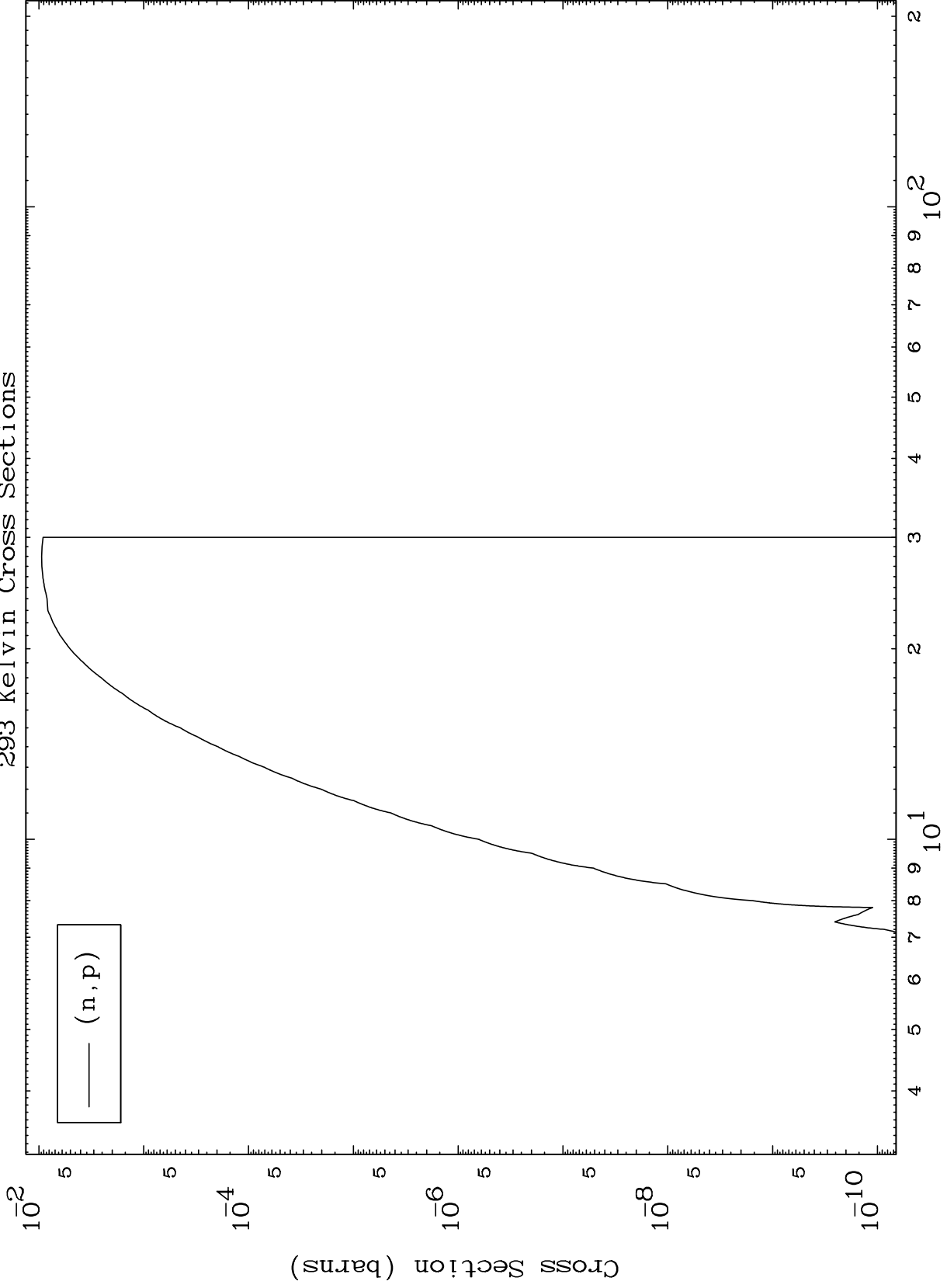
92-U -243



MAT 9252

(n,p) Levels  
293 Kelvin Cross Sections

92-U -243



12

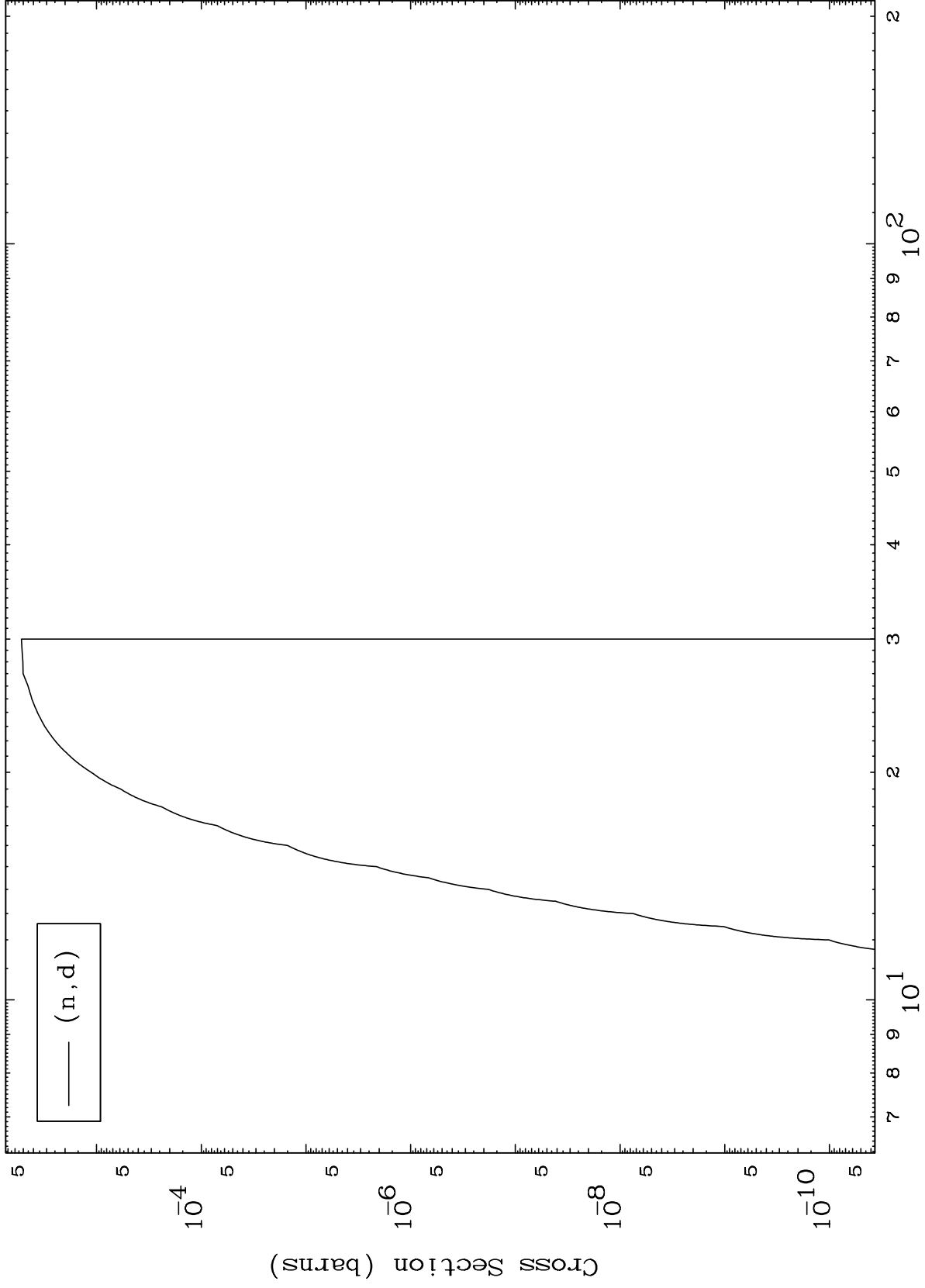
Incident Energy (MeV)

92-U -243

MAT 9252

(n,d) Levels  
293 Kelvin Cross Sections

92-U -243



13

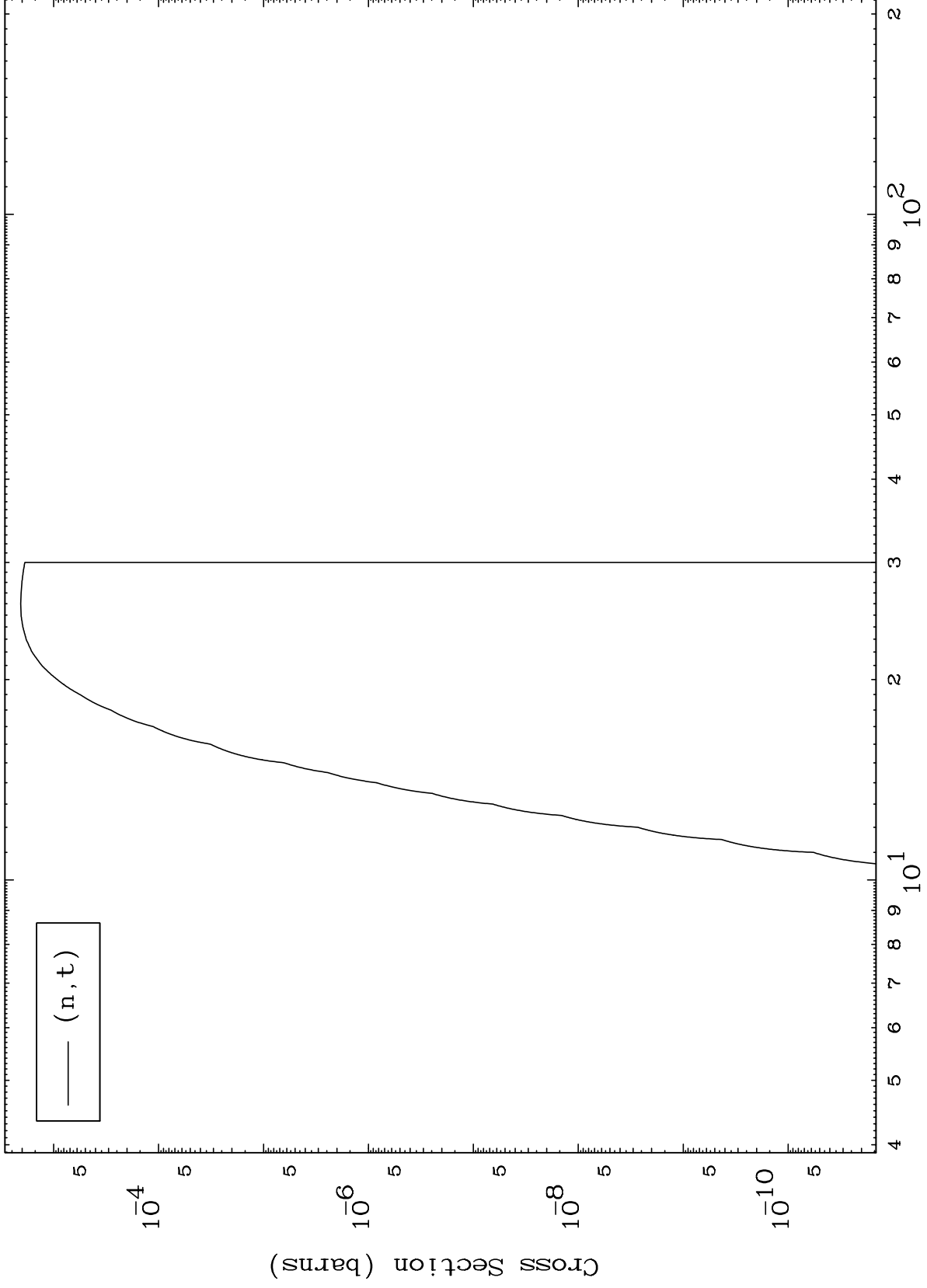
Incident Energy (MeV)

92-U -243

MAT 9252

(n,t) Levels  
293 Kelvin Cross Sections

92-U -243



14

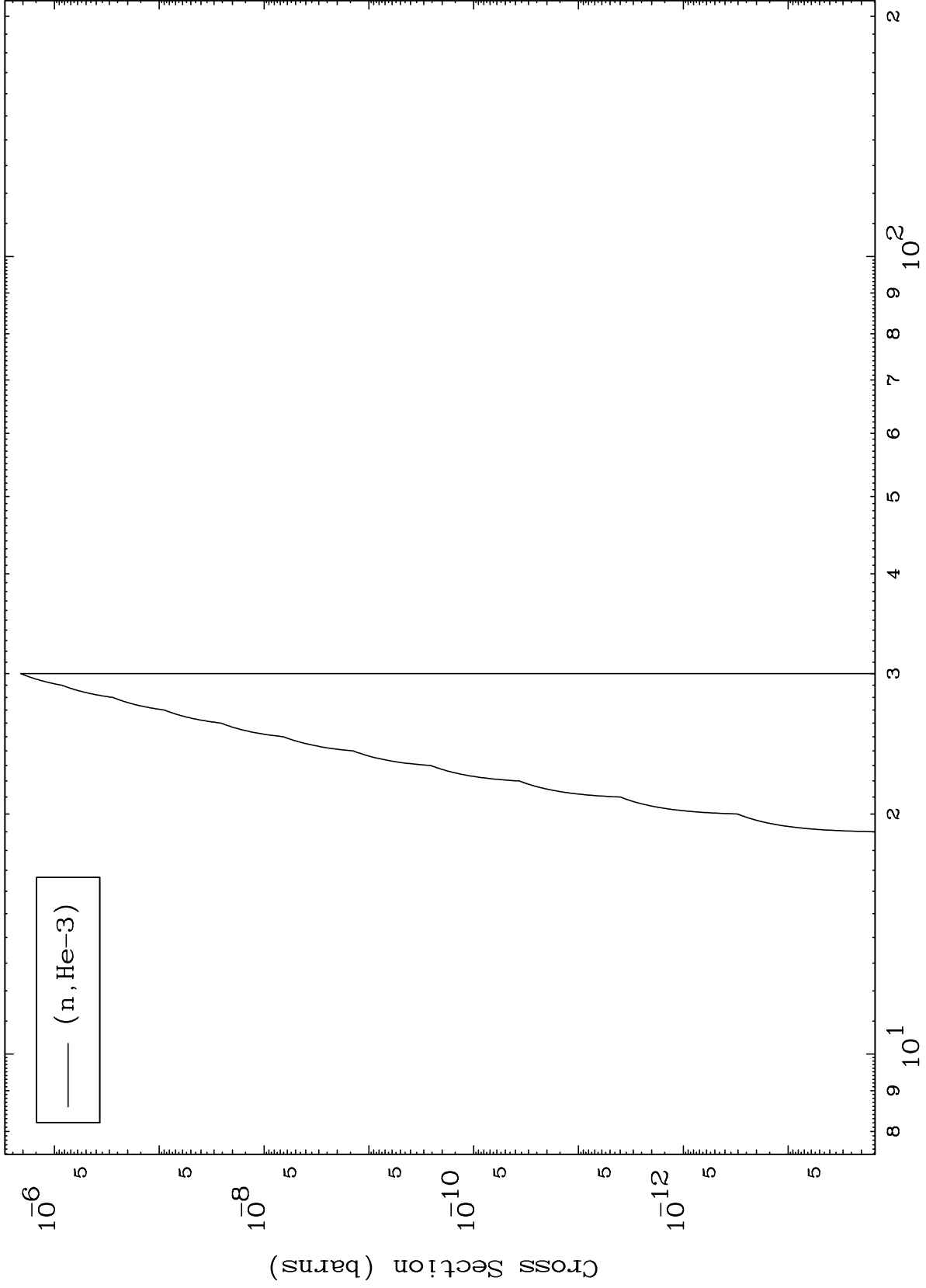
Incident Energy (MeV)

92-U -243

MAT 9252

(n,He3) Levels  
293 Kelvin Cross Sections

92-U -243



15

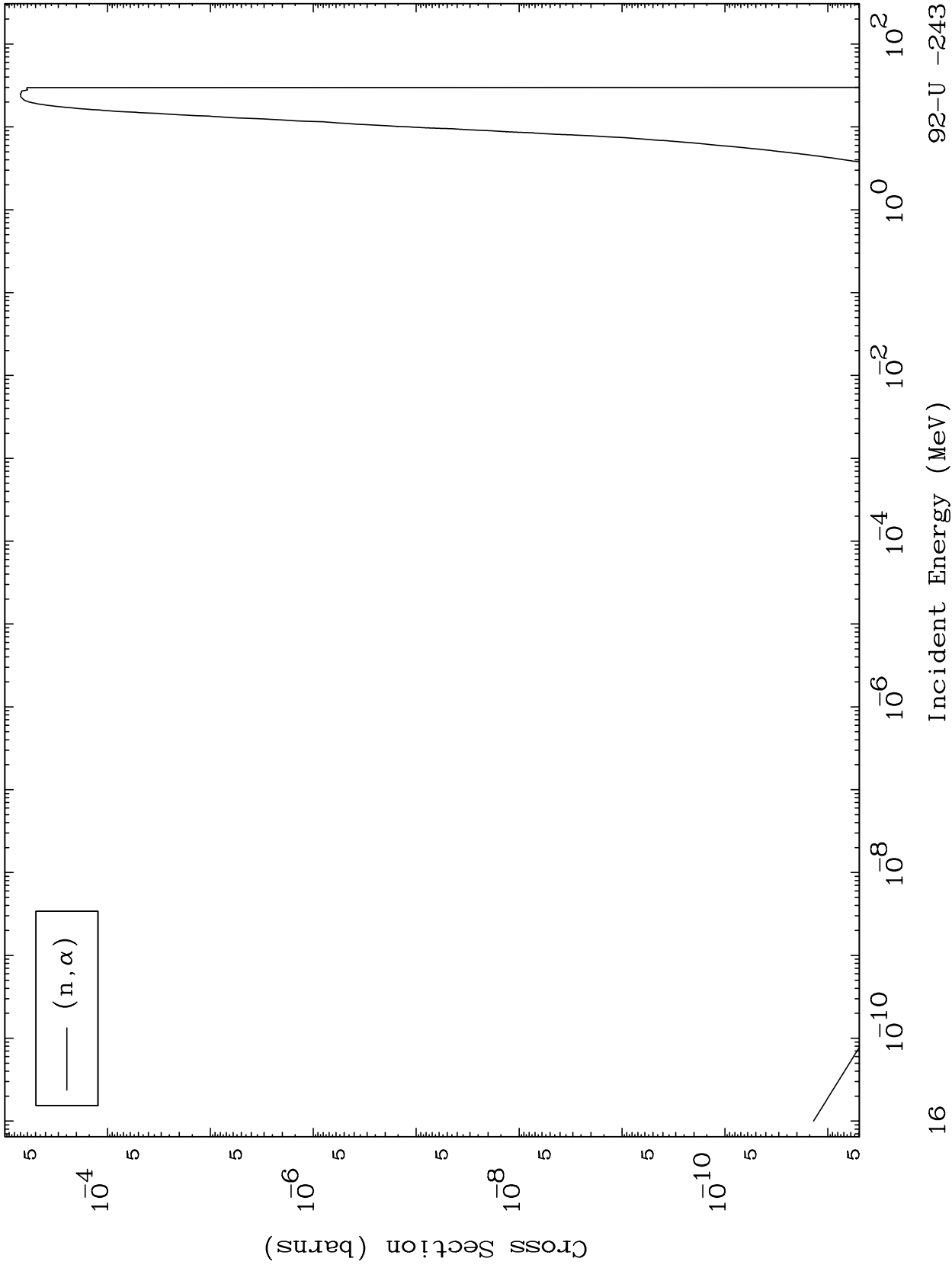
Incident Energy (MeV)

92-U -243

MAT 9252

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

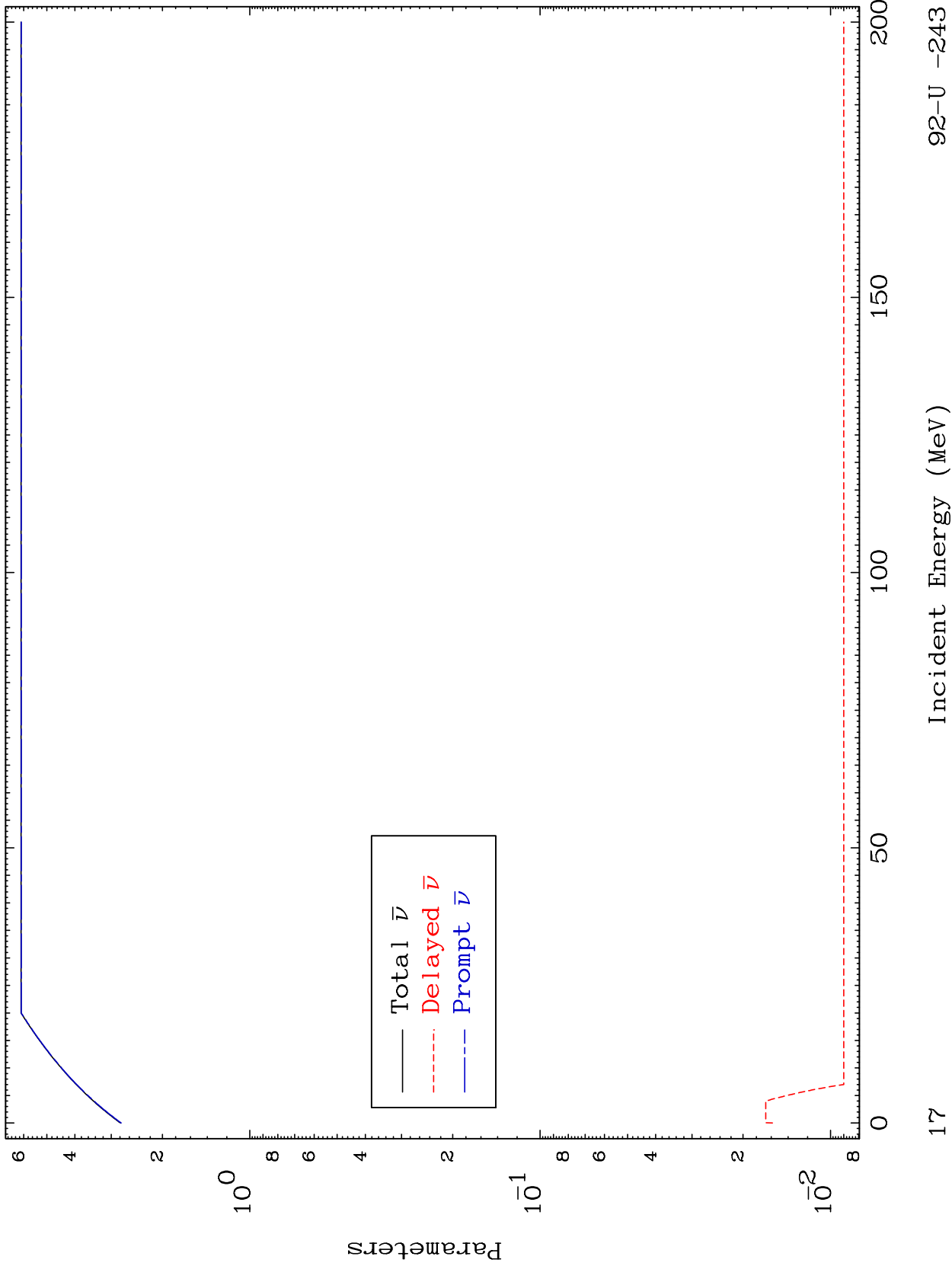
92-U -243



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Energy Release  
Parameters

92-U -243



17

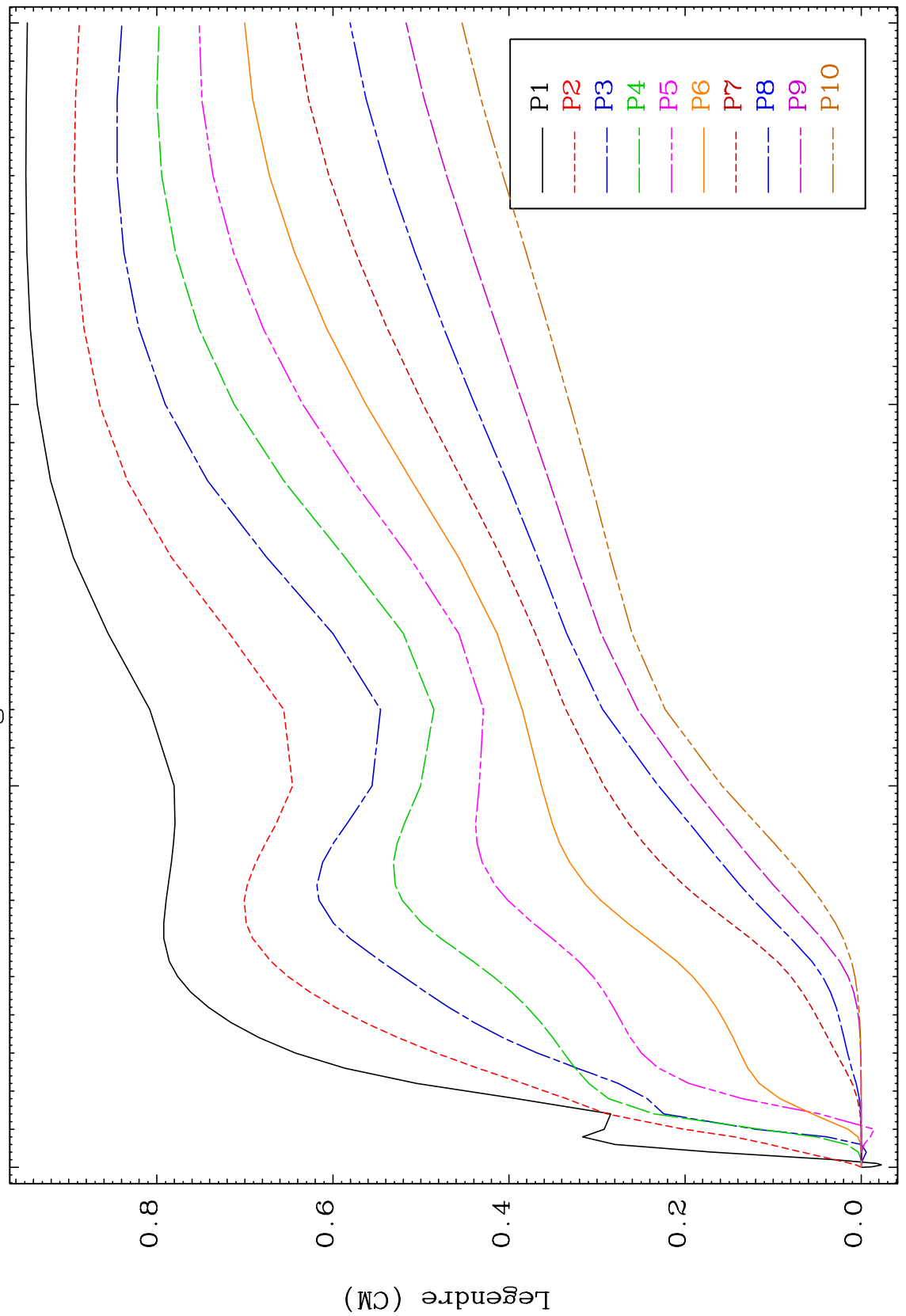
Incident Energy (MeV)

92-U -243

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92-U -243

Elastic Legendre Coefficients



92-U -243

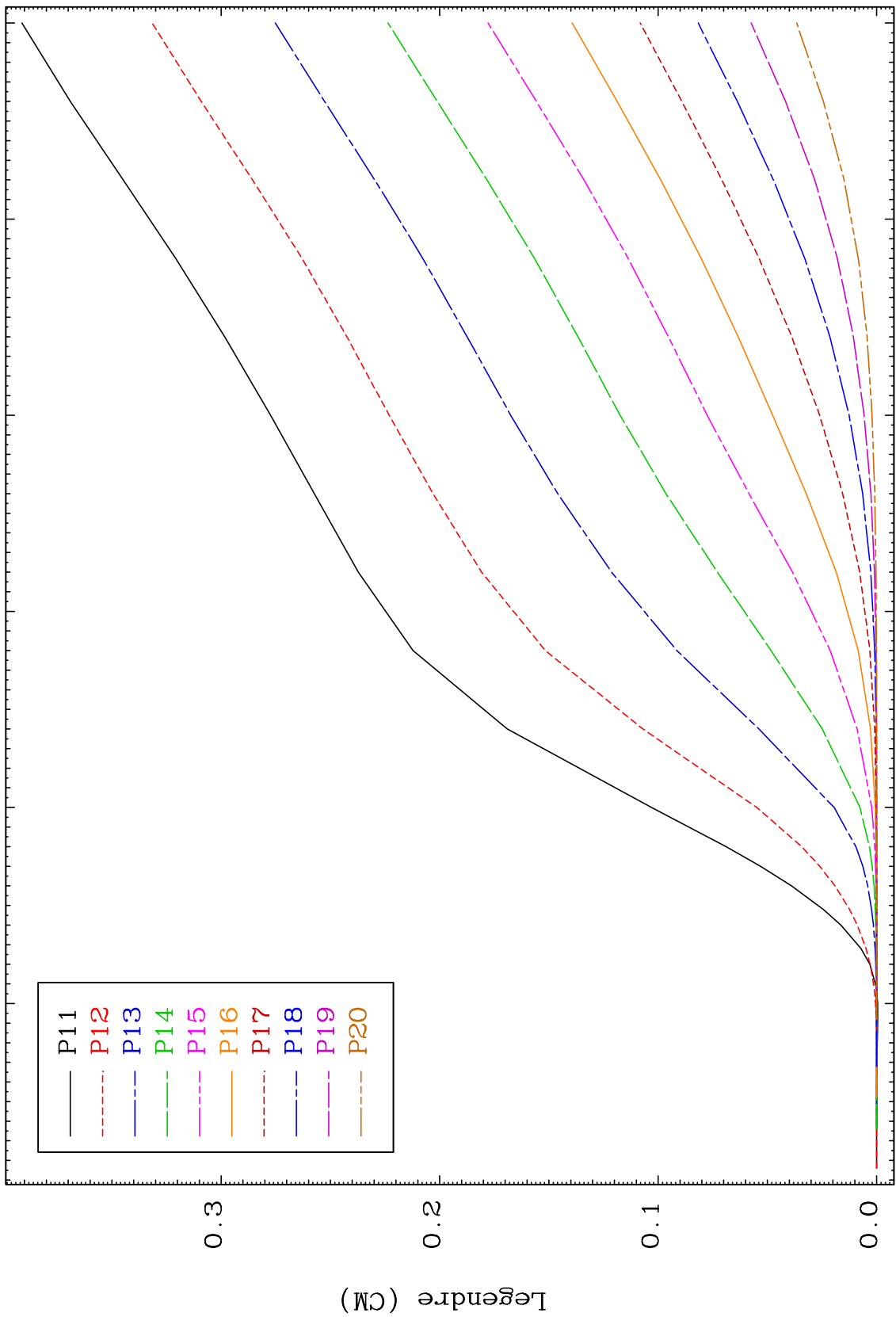
Incident Energy (MeV)

18

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Elastic Legendre Coefficients

92-U -243



19

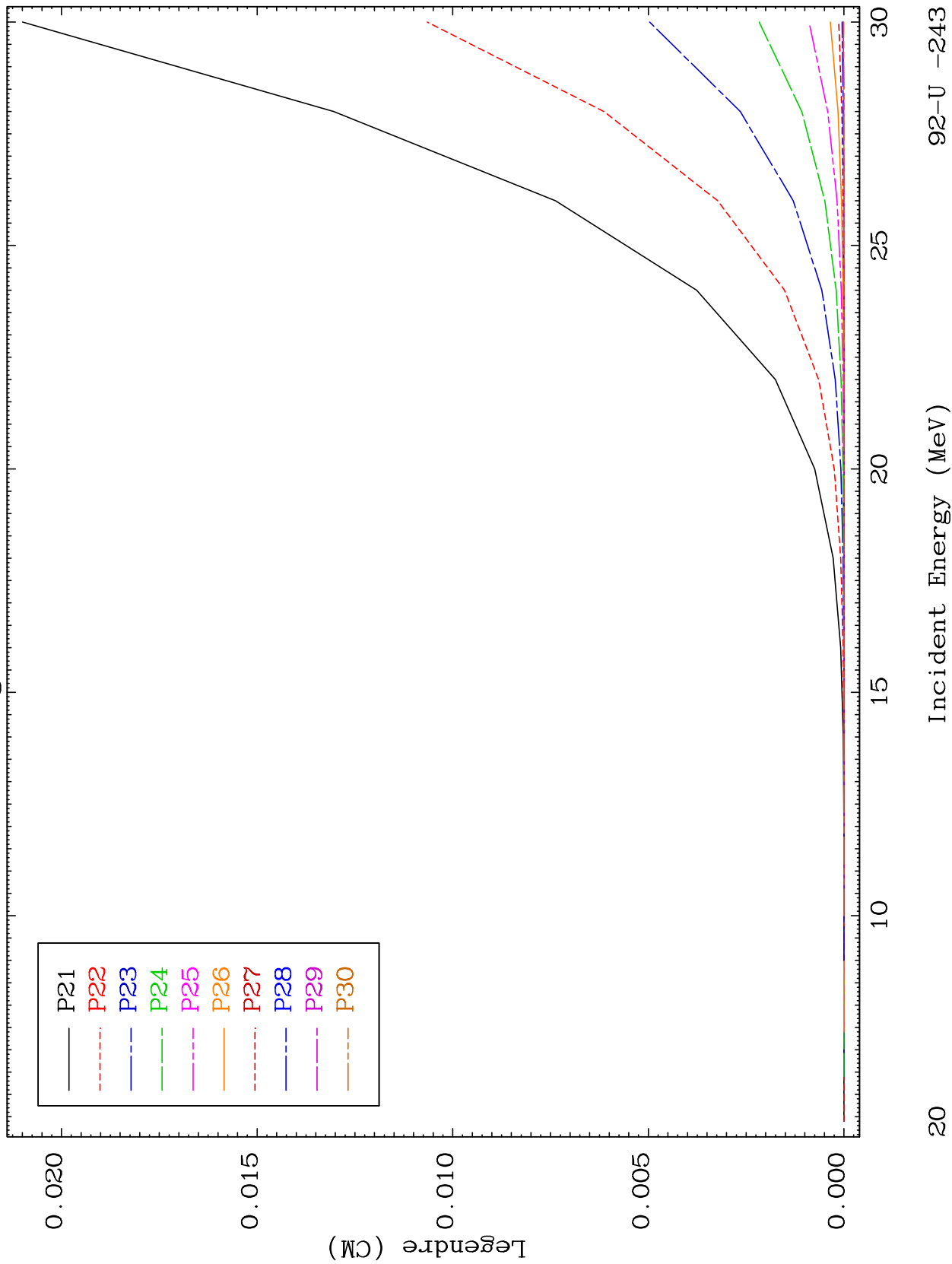
Incident Energy (MeV)

92-U -243

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### Elastic Legendre Coefficients

92-U -243



20

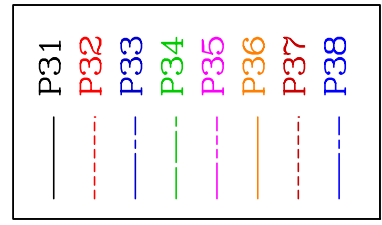
Incident Energy (MeV)

92-U -243

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Elastic Legendre Coefficients

92-U -243



$\times 10^{16}$

Legendre (CM)

1.0

0.5

0.0

15

20

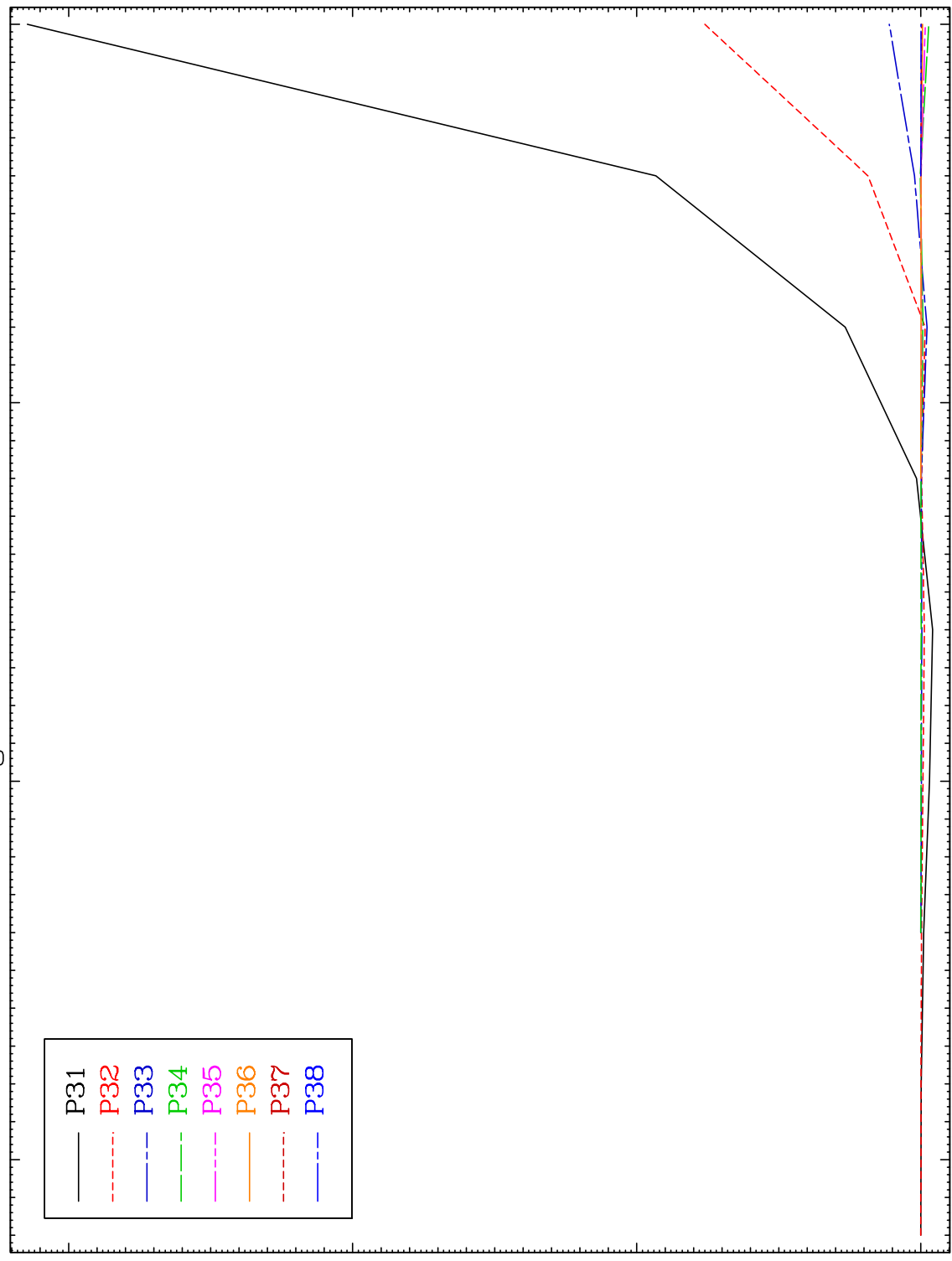
25

30

21

Incident Energy (MeV)

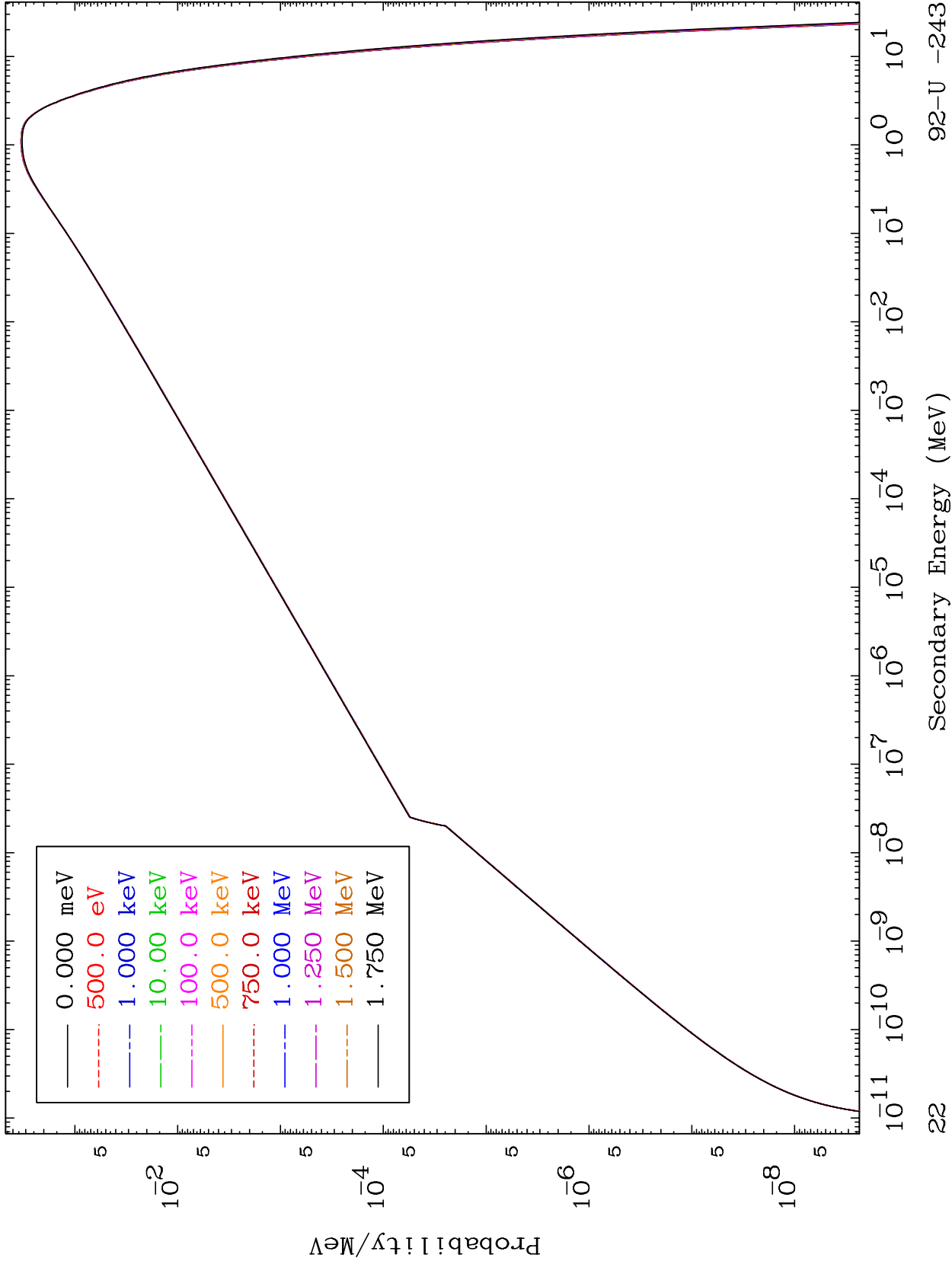
92-U -243



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### Fission Energy Distributions

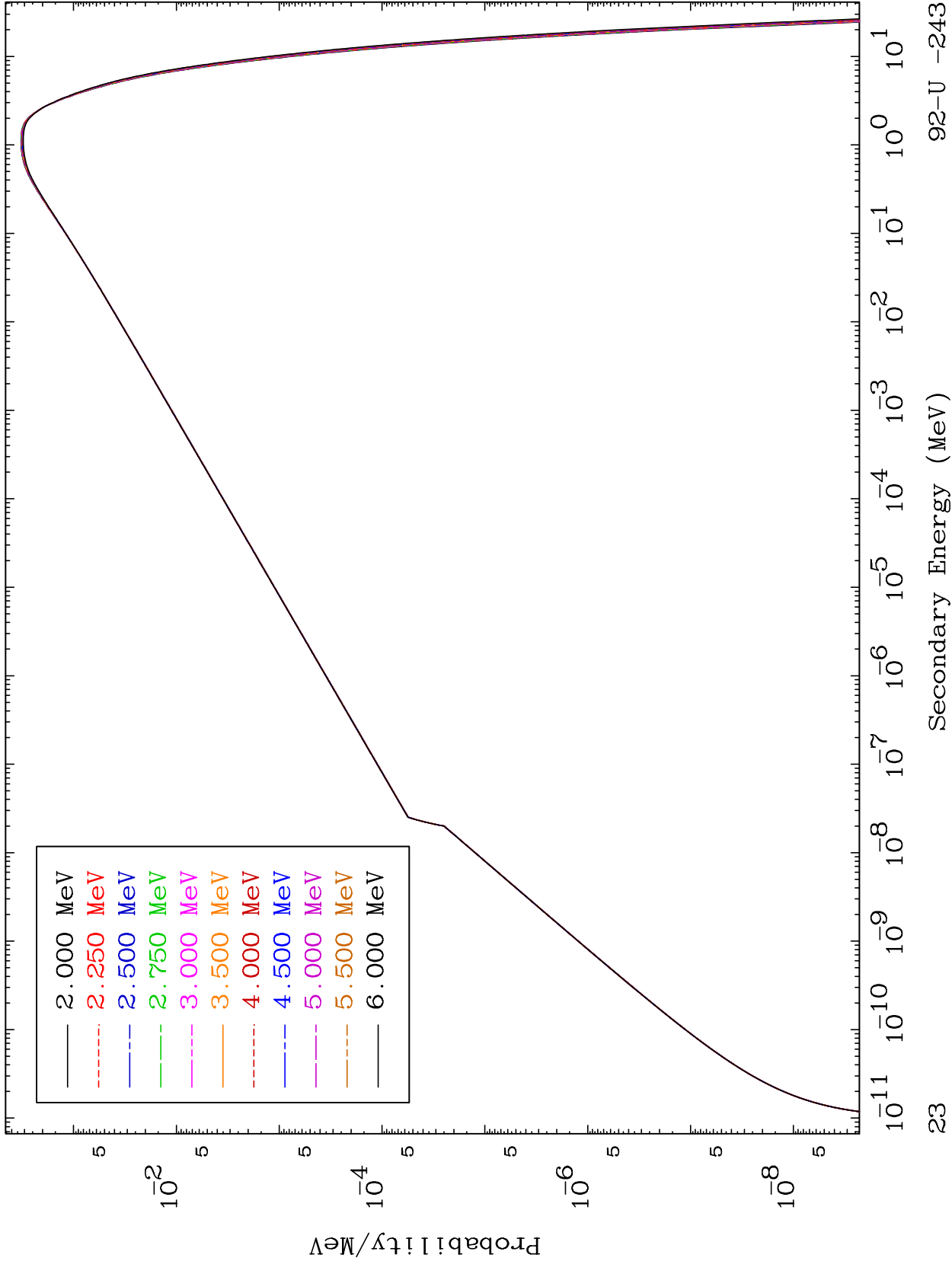
92-U -243



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### Fission Energy Distributions

92-U -243



92-U -243

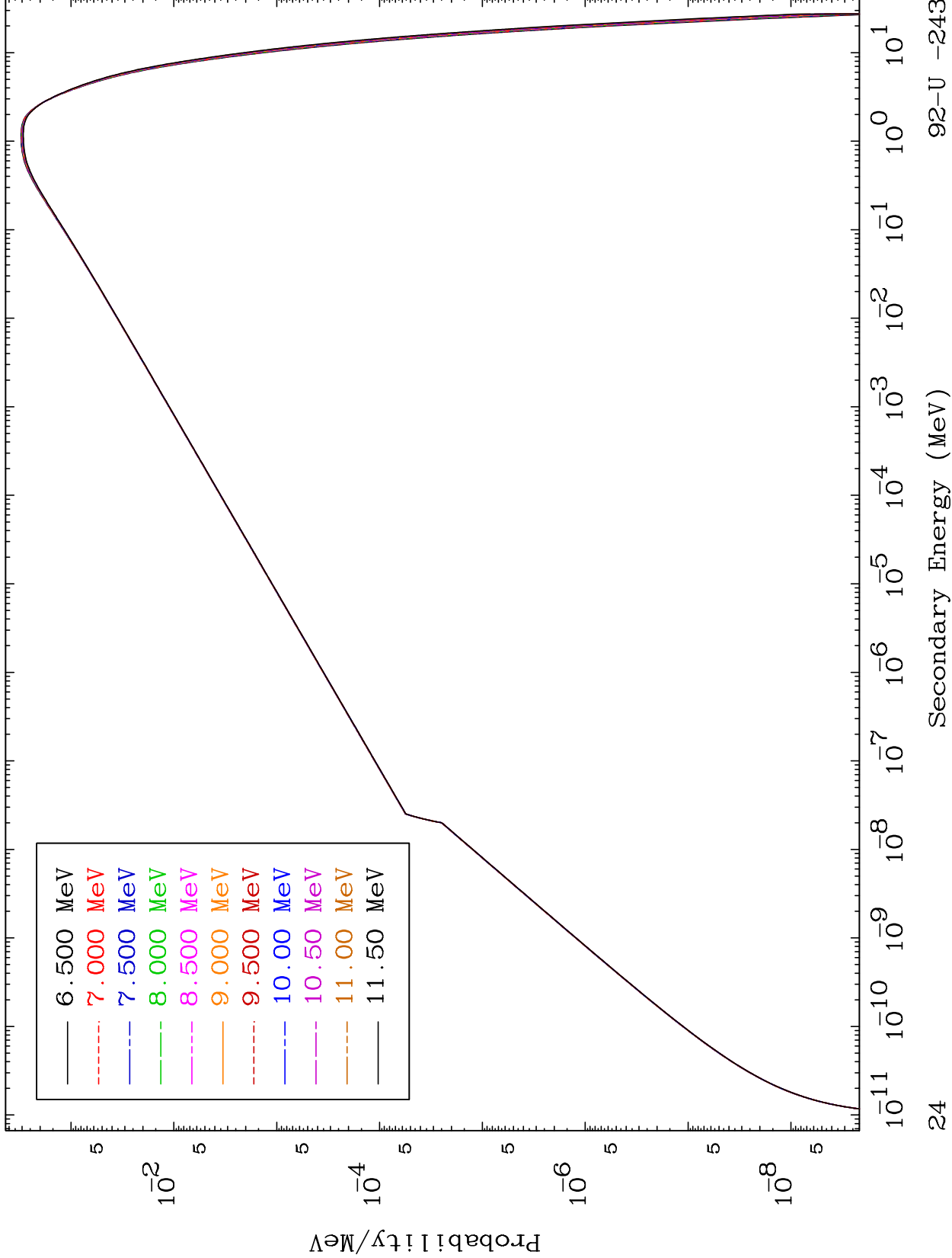
Secondary Energy (MeV)

23

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### Fission Energy Distributions

92-U -243



92-U -243

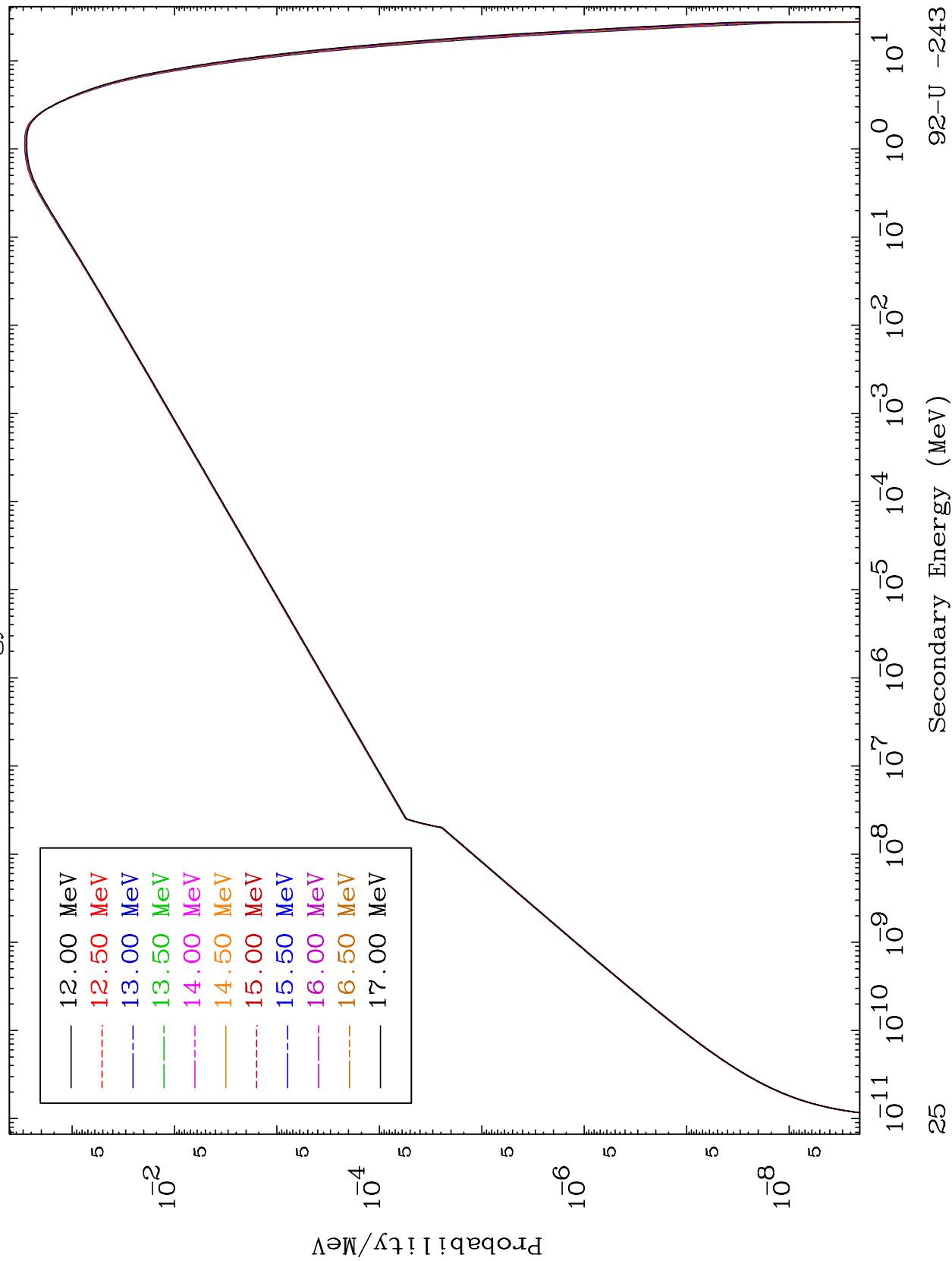
Secondary Energy (MeV)

24

MAT 9252

### Fission Energy Distributions

92-U -243



92-U -243

Secondary Energy (MeV)

25

MAT 9252

### Fission Energy Distributions

92-U -243

