

Program EVALPLOT
(Version 2021-1)

by

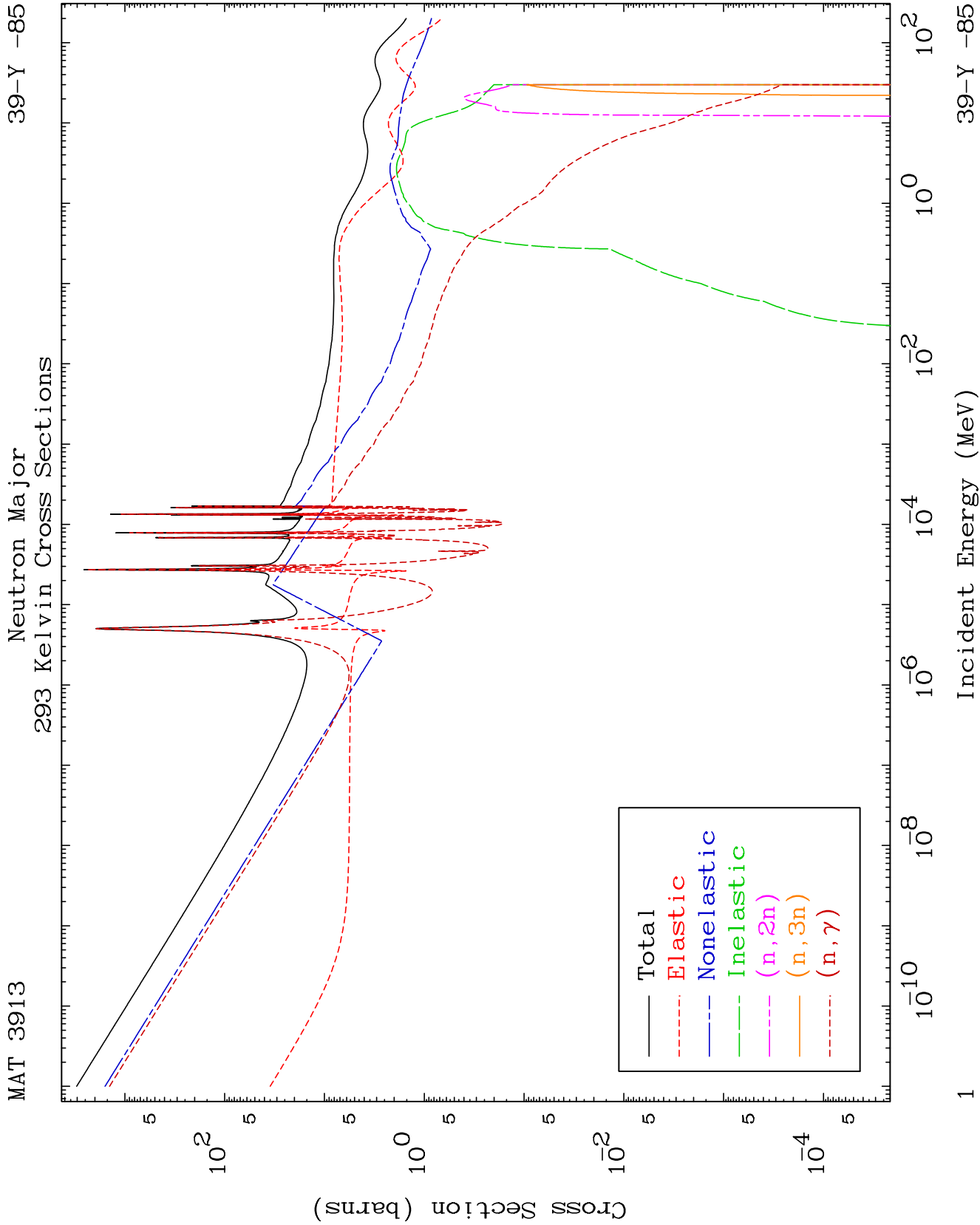
Dermott E. Cullen
(Present Contact Information)

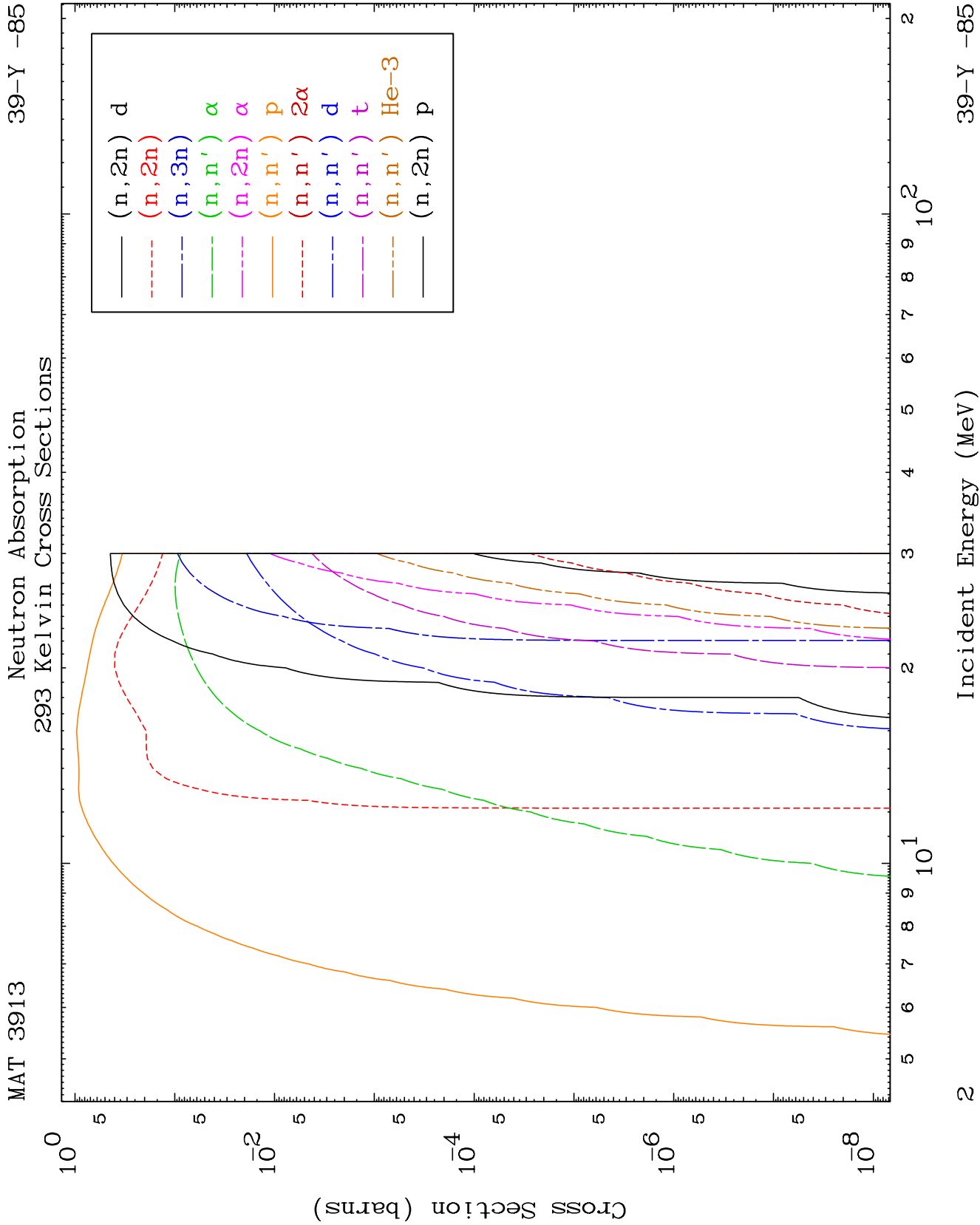
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Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

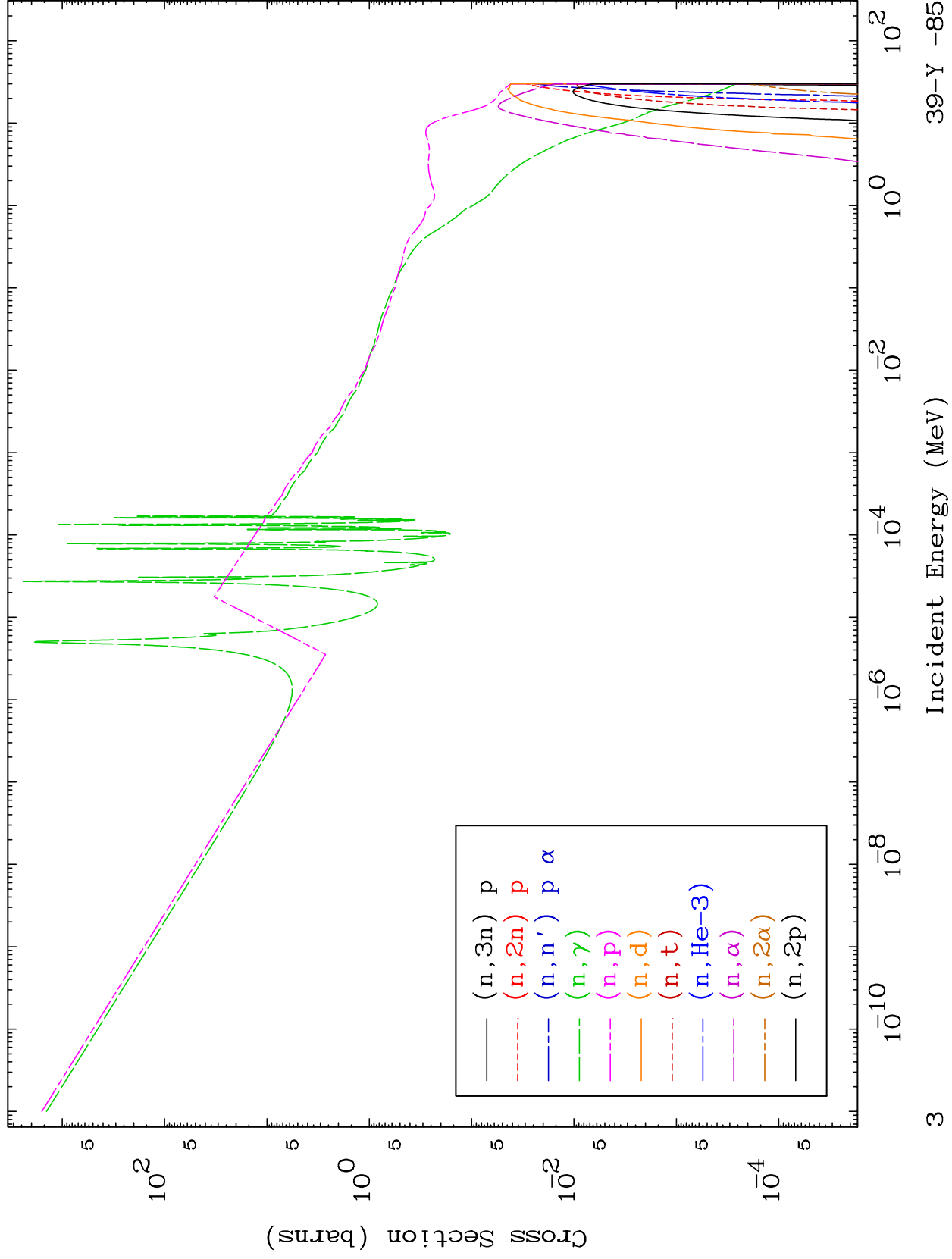


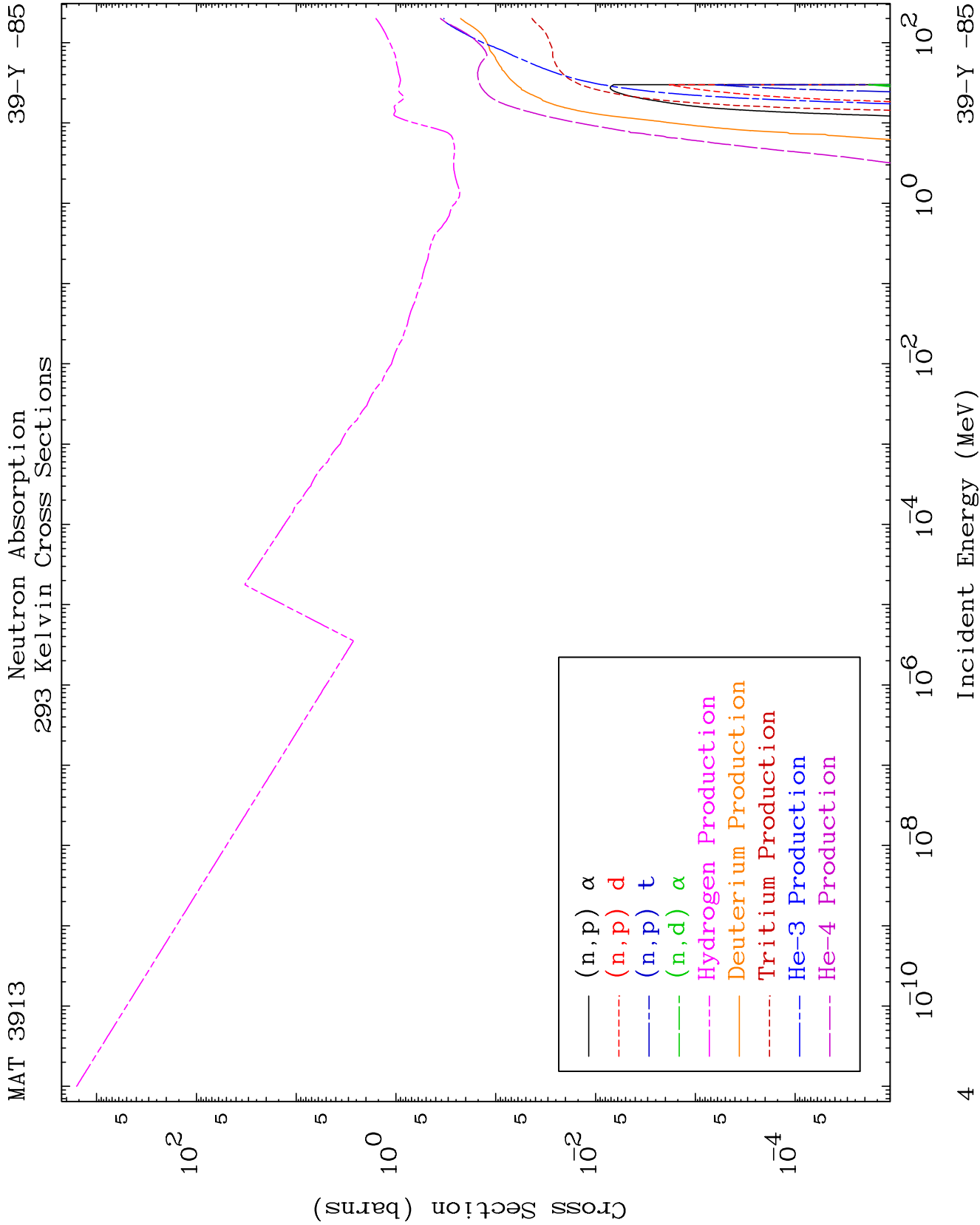


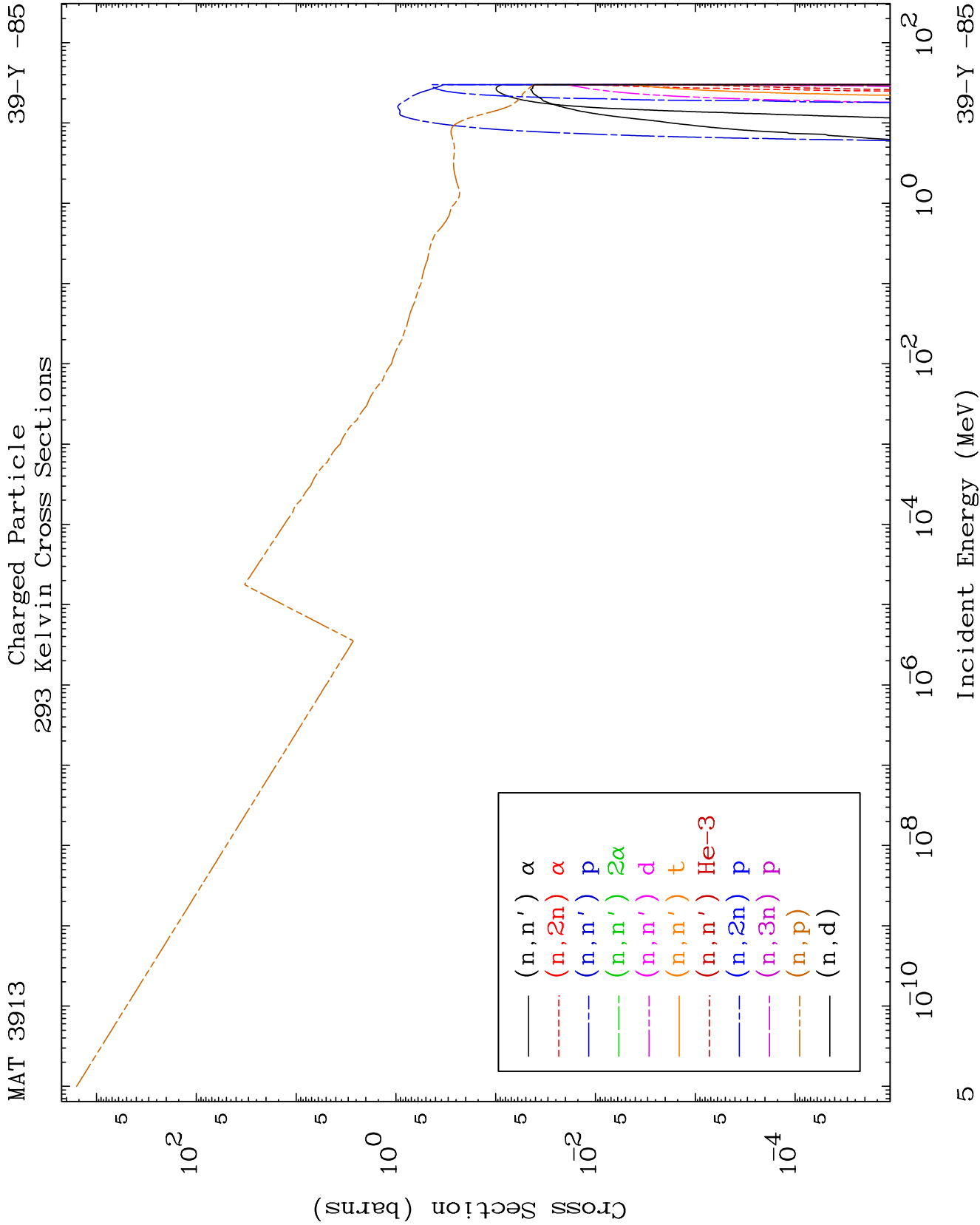
MAT 3913

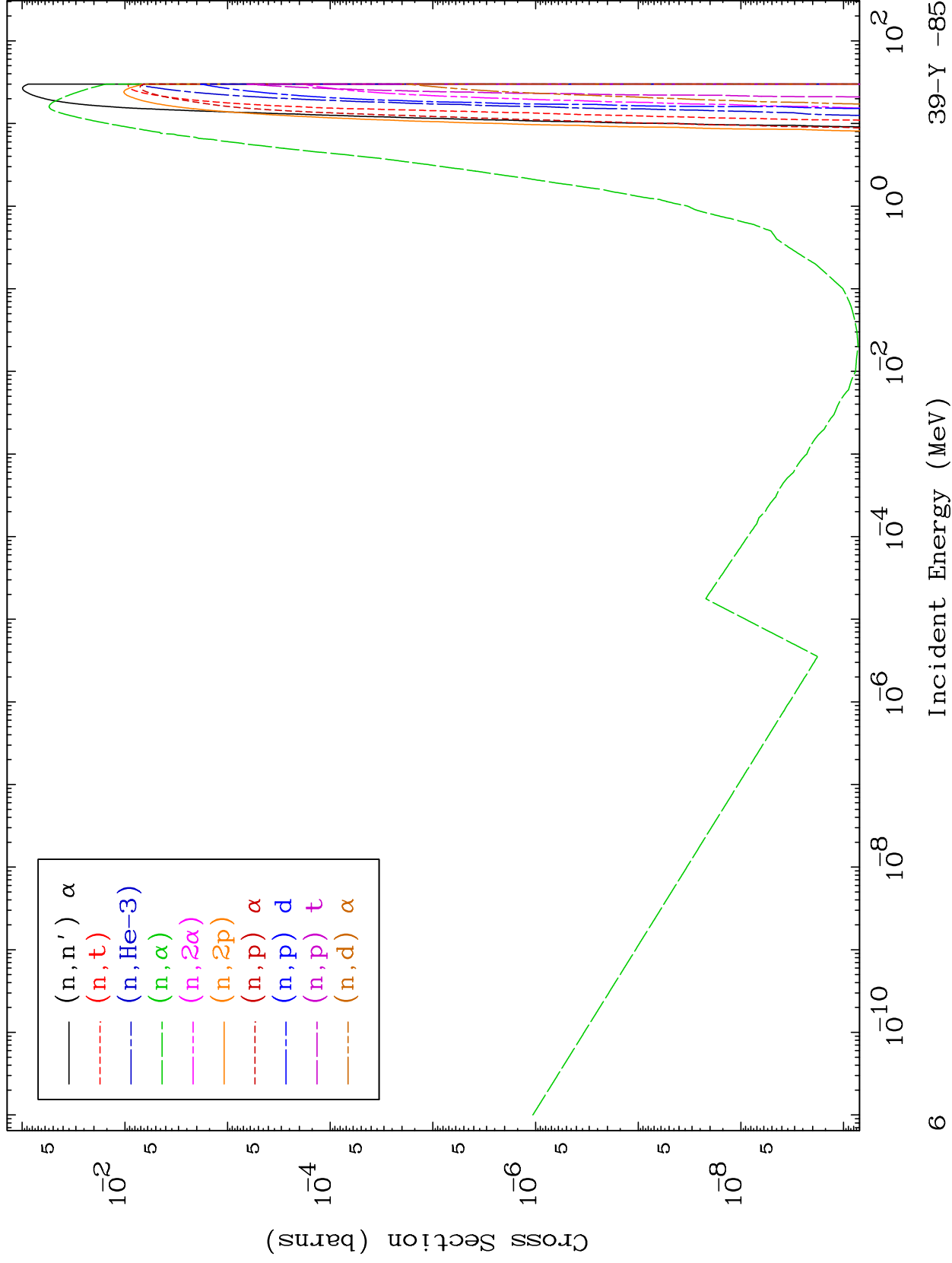
Neutron Absorption
293 Kelvin Cross Sections

39-Y -85





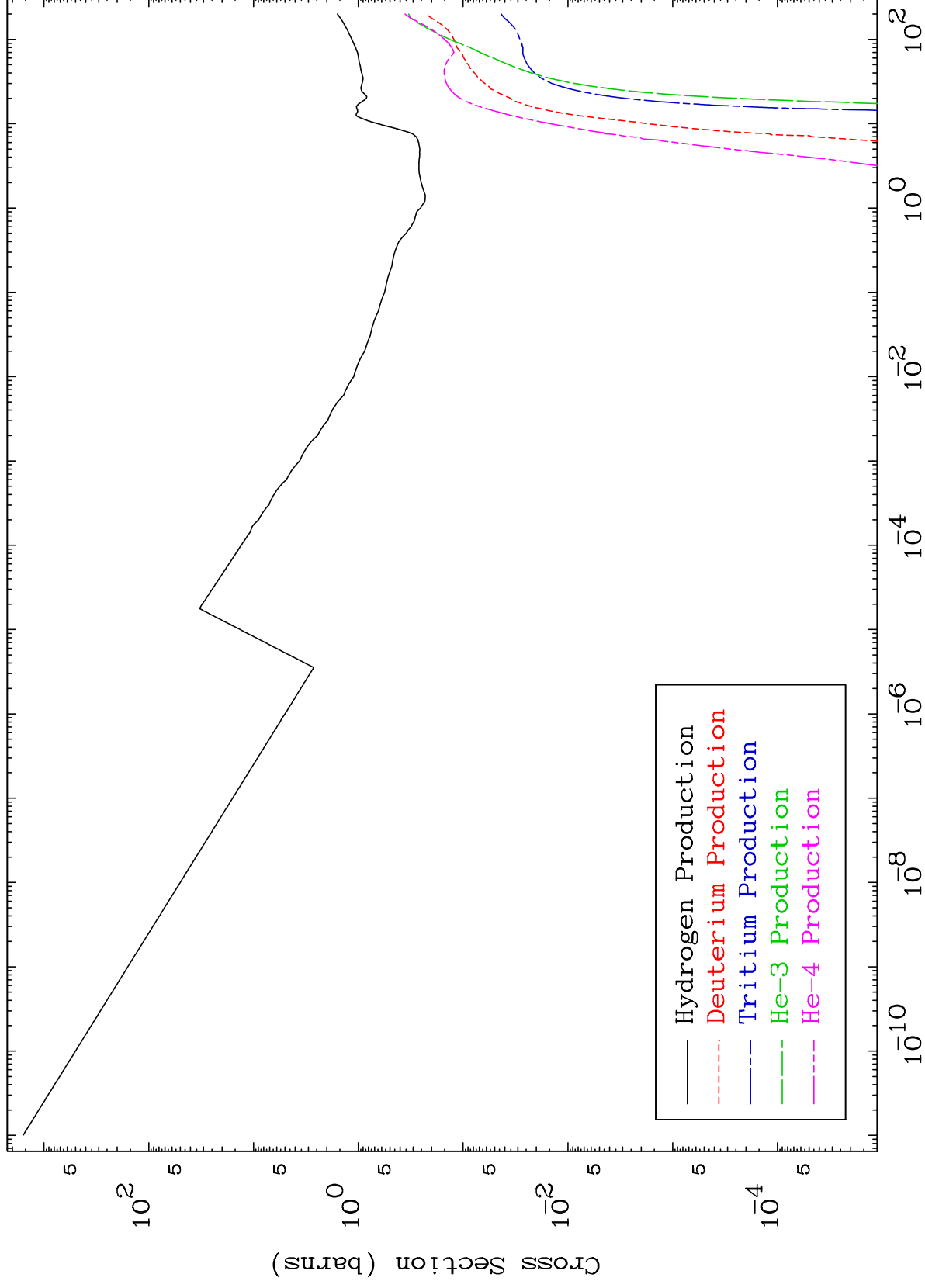


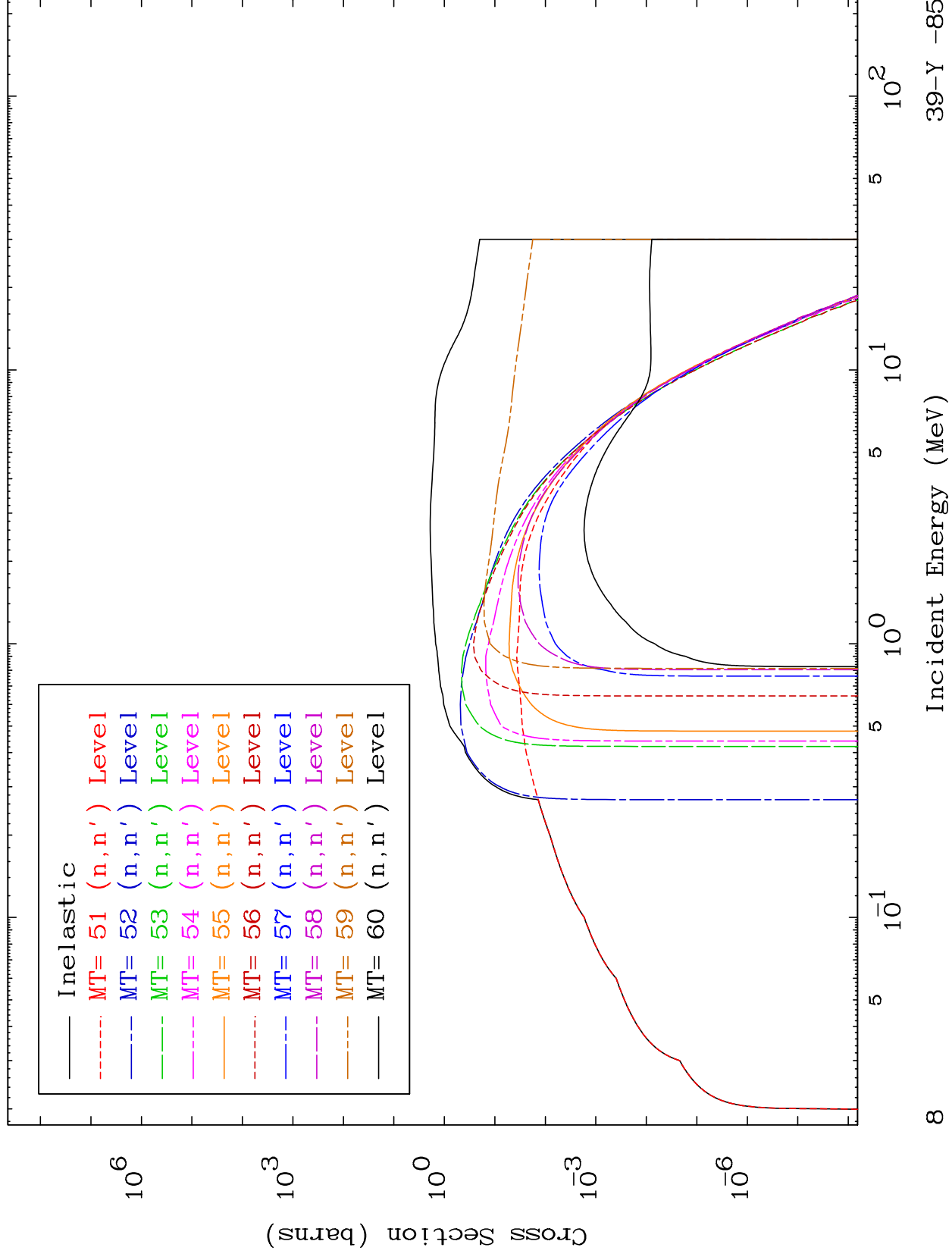


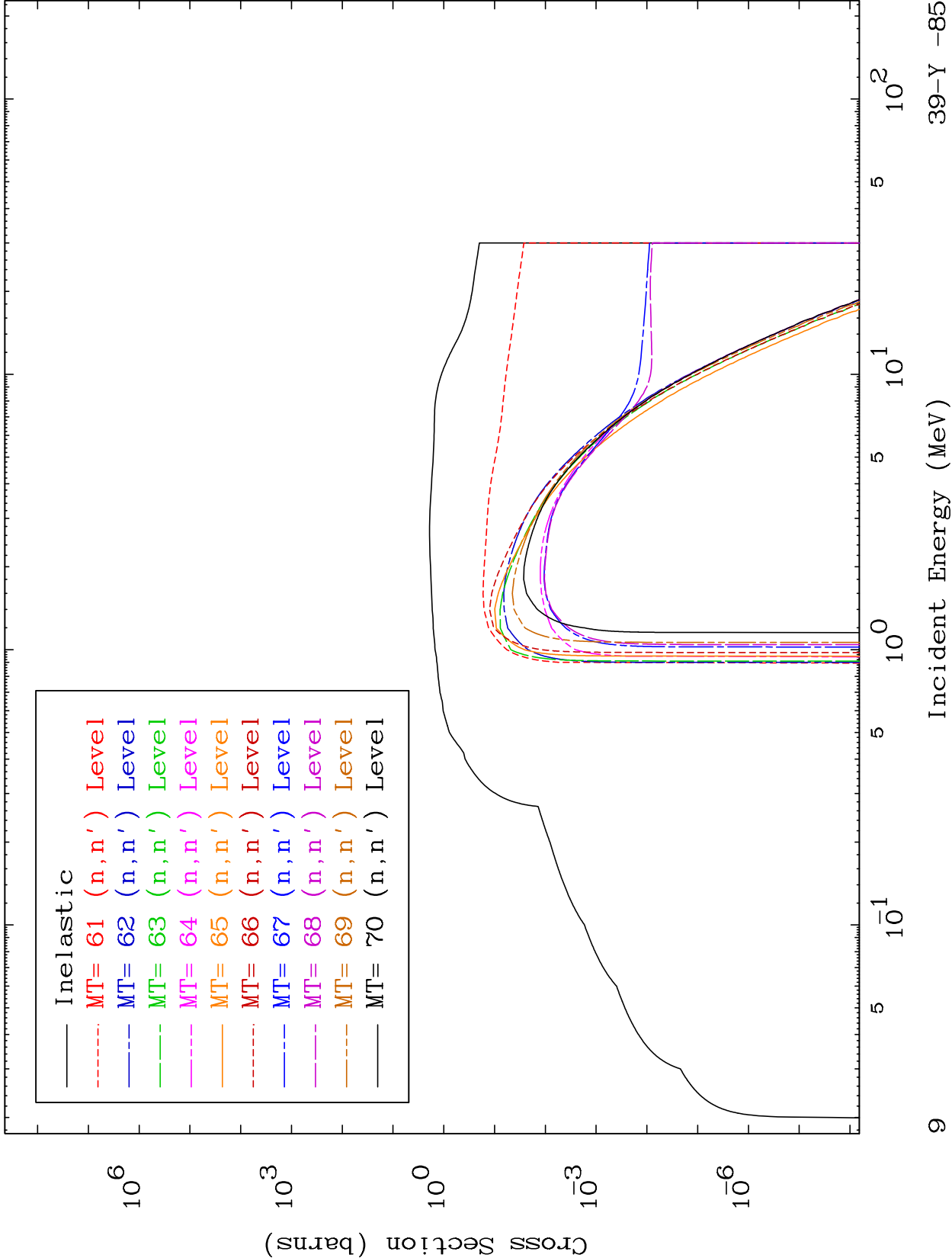
MAT 3913

Particle Production
293 Kelvin Cross Sections

39-Y -85



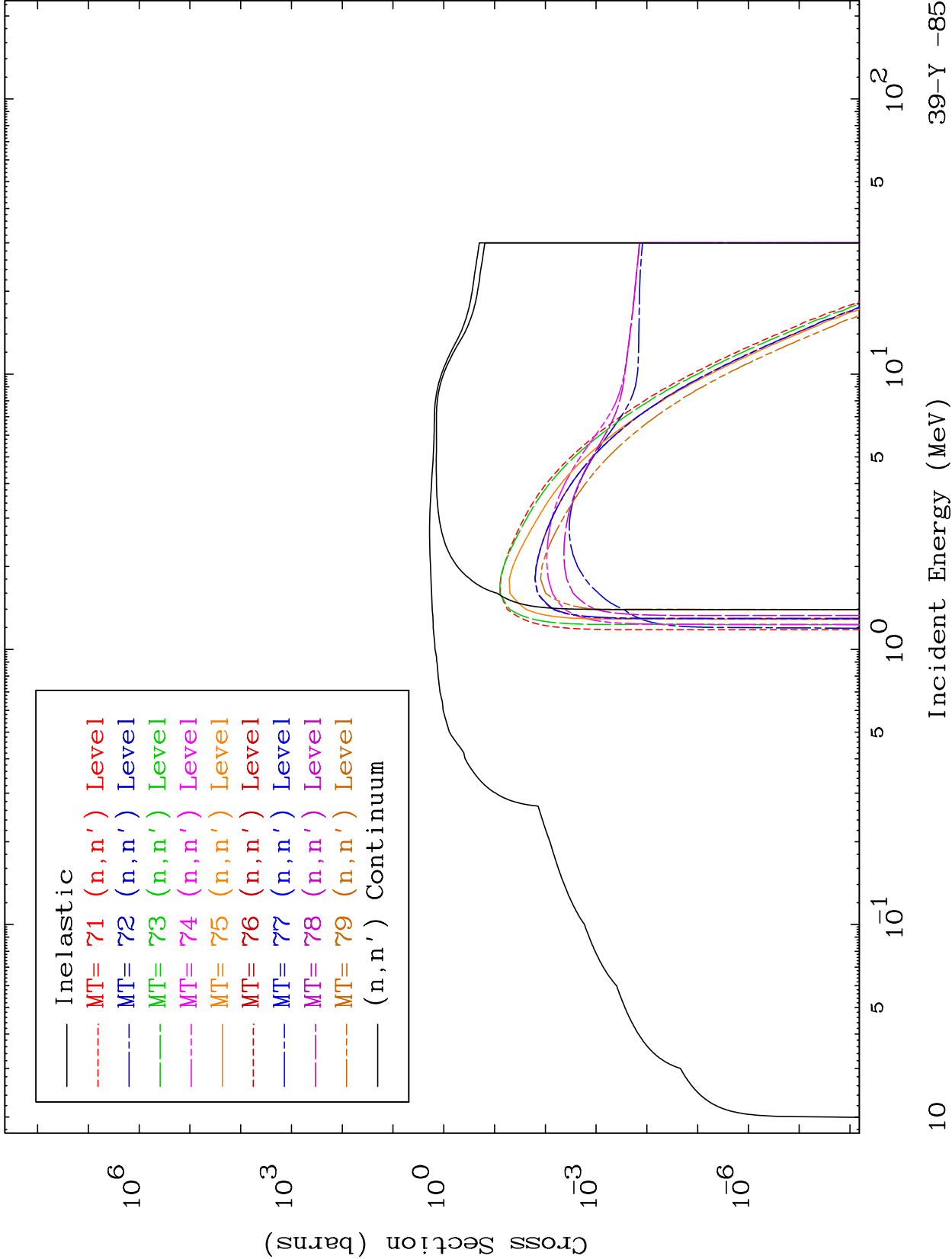




MAT 3913

(n,n') Levels
293 Kelvin Cross Sections

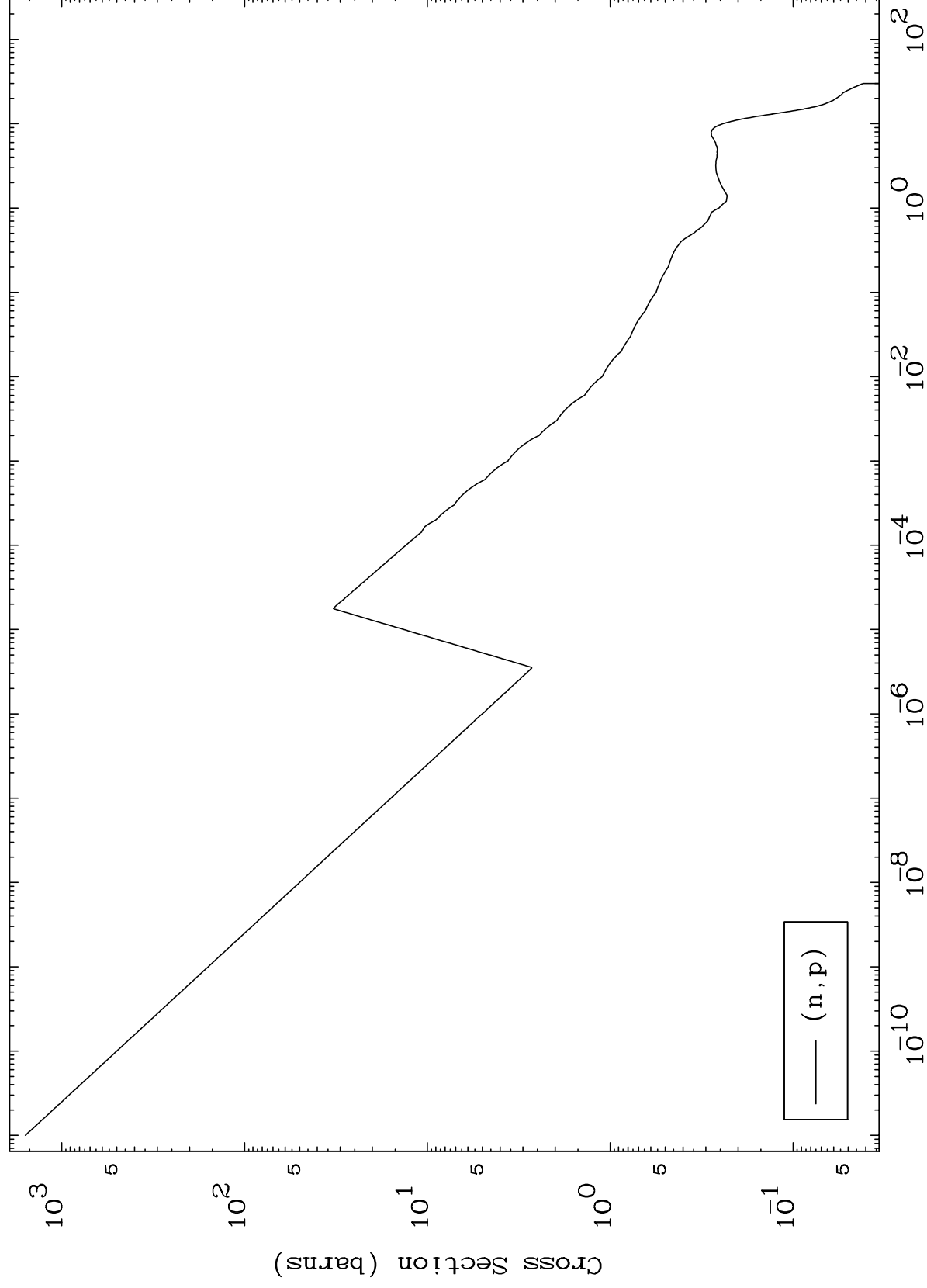
39-Y -85



MAT 3913

(n,p) Levels
293 Kelvin Cross Sections

39-Y -85



— (n,p)

11

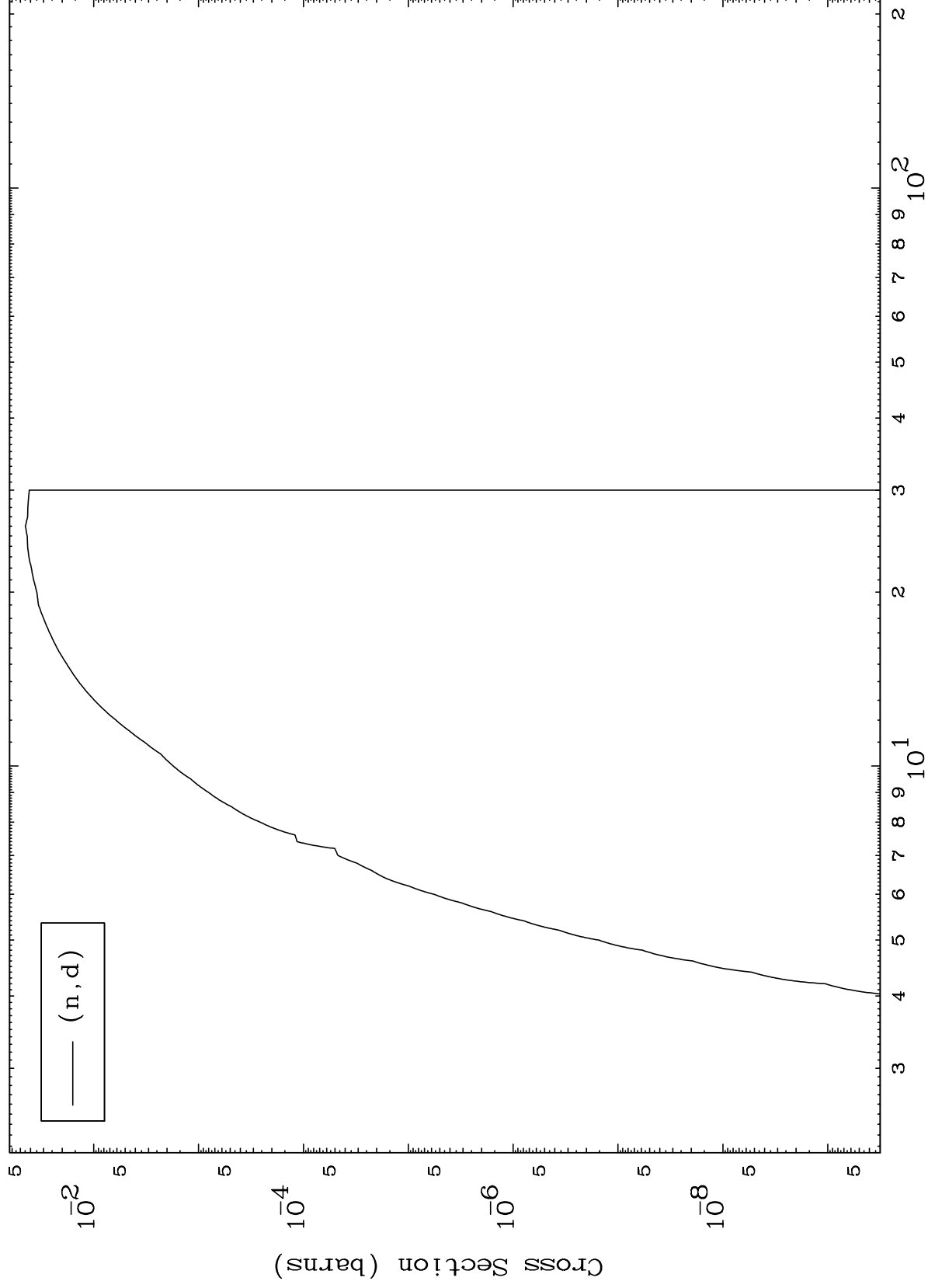
Incident Energy (MeV)

39-Y -85

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39-Y -85

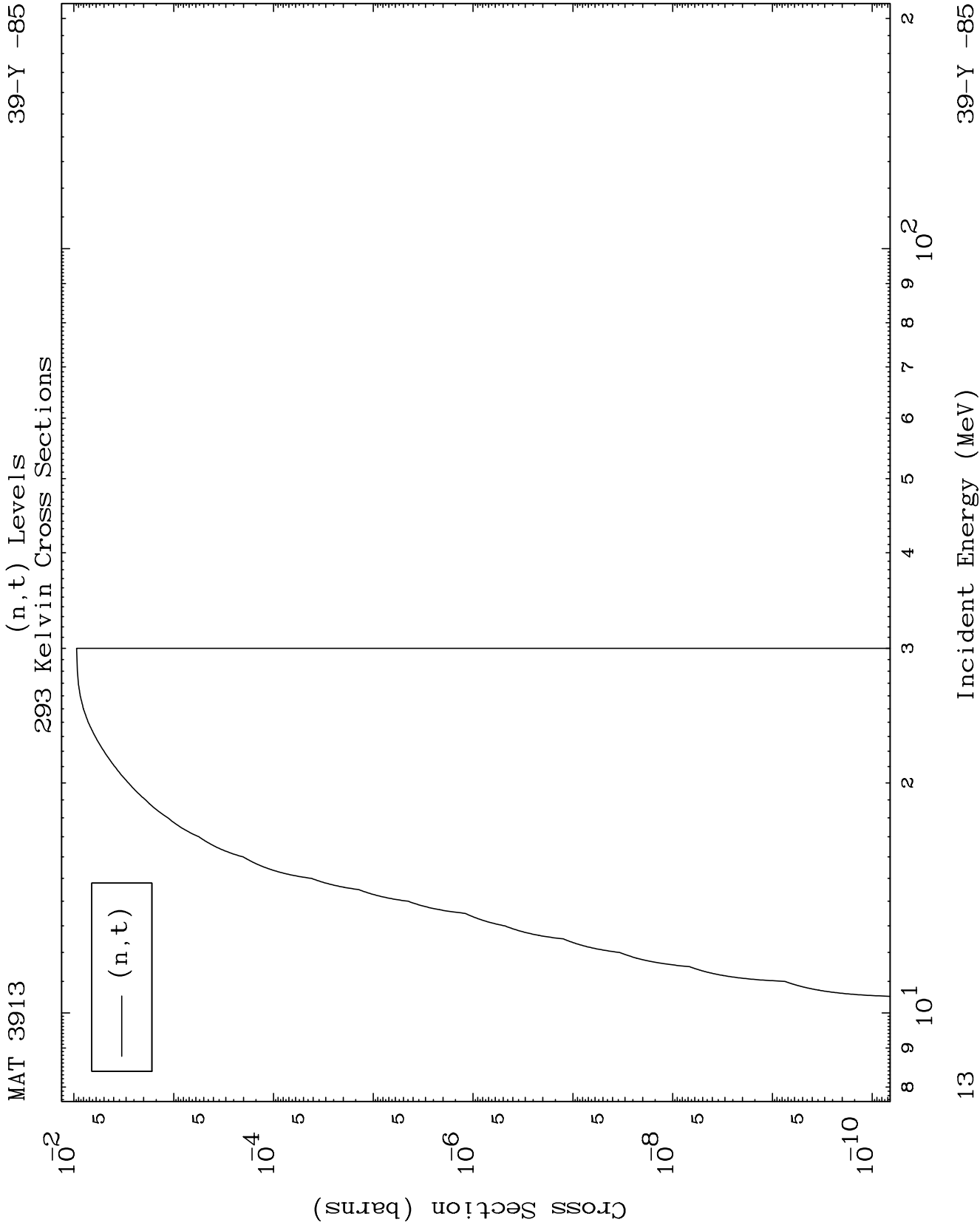
(n,d) Levels
293 Kelvin Cross Sections

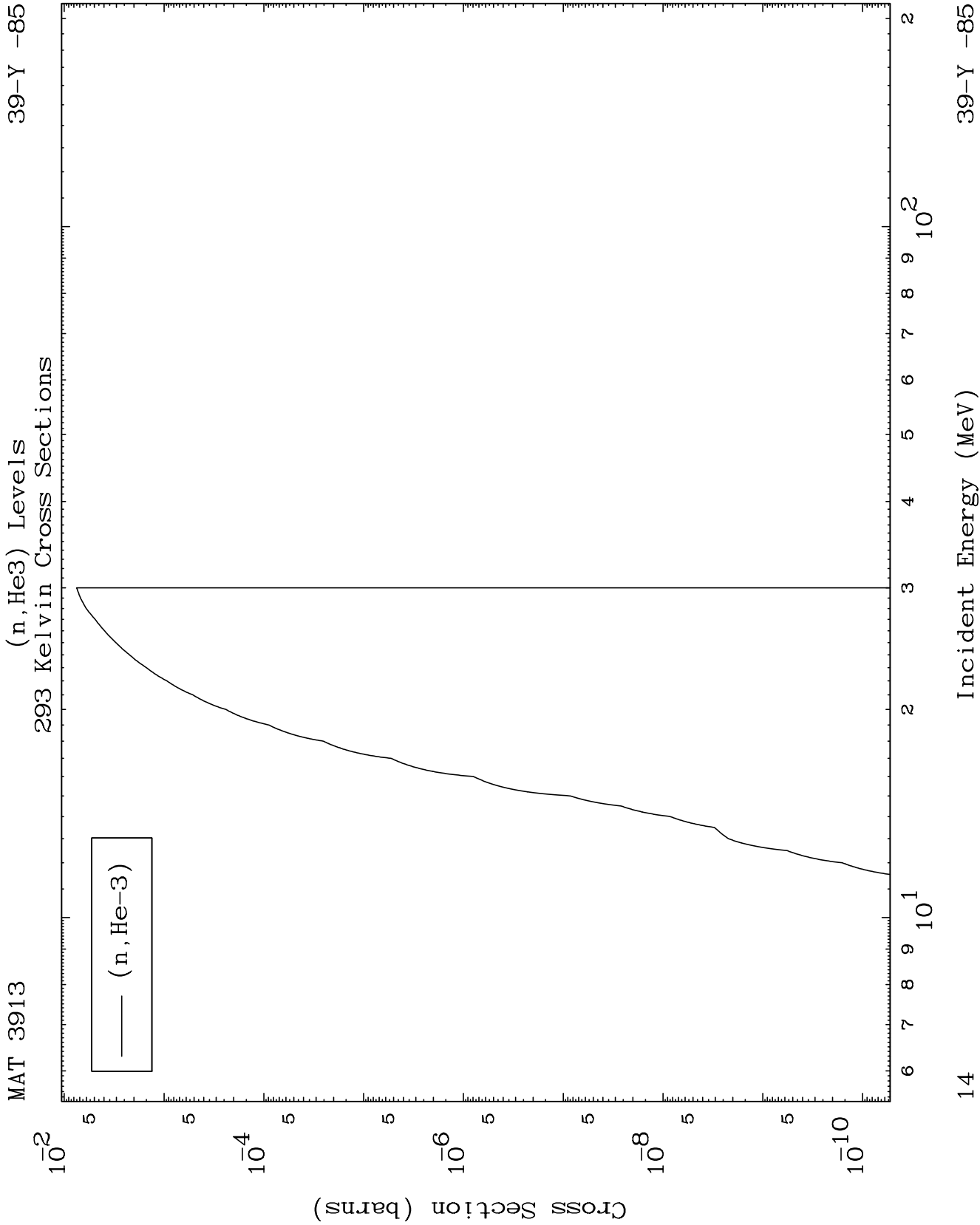


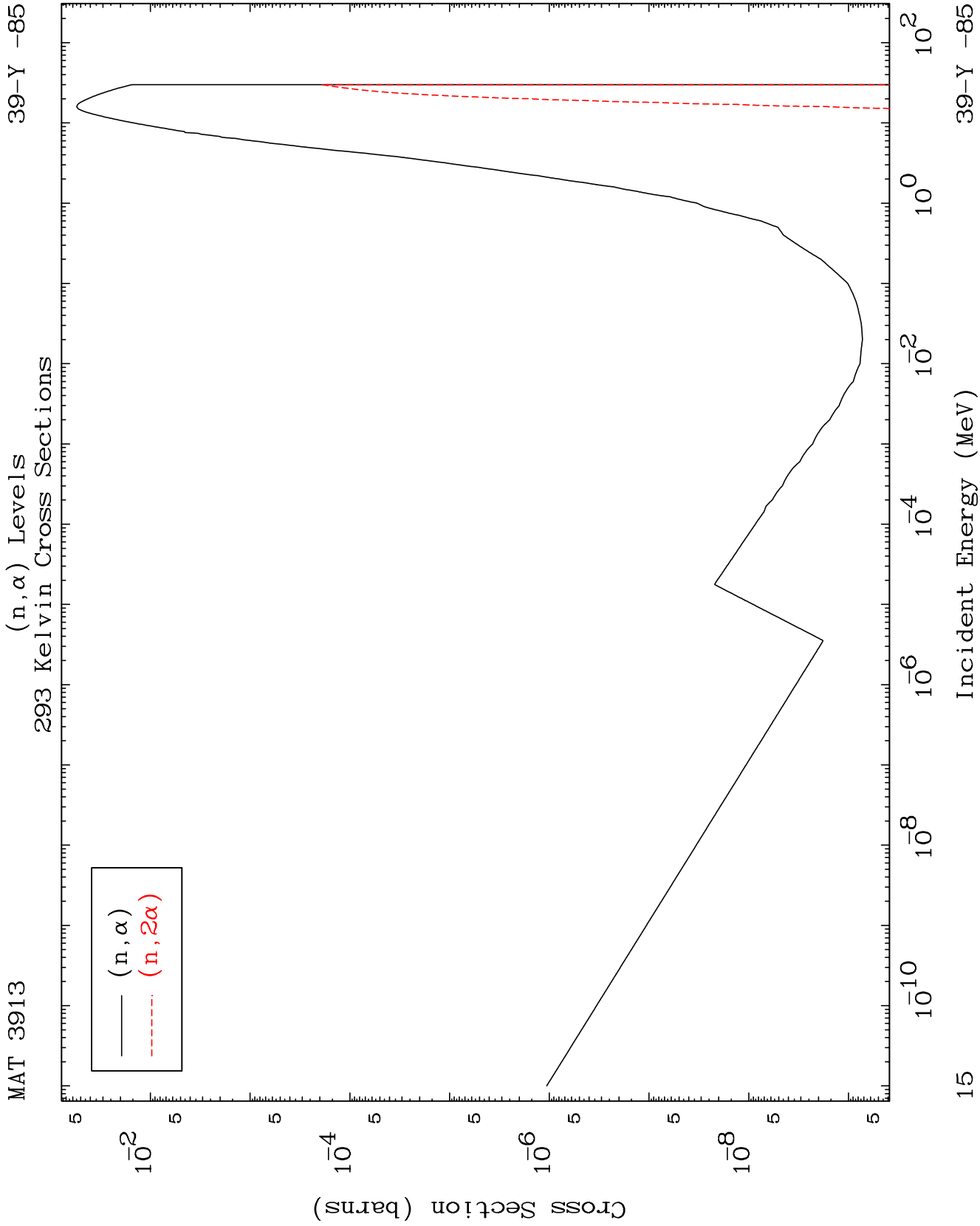
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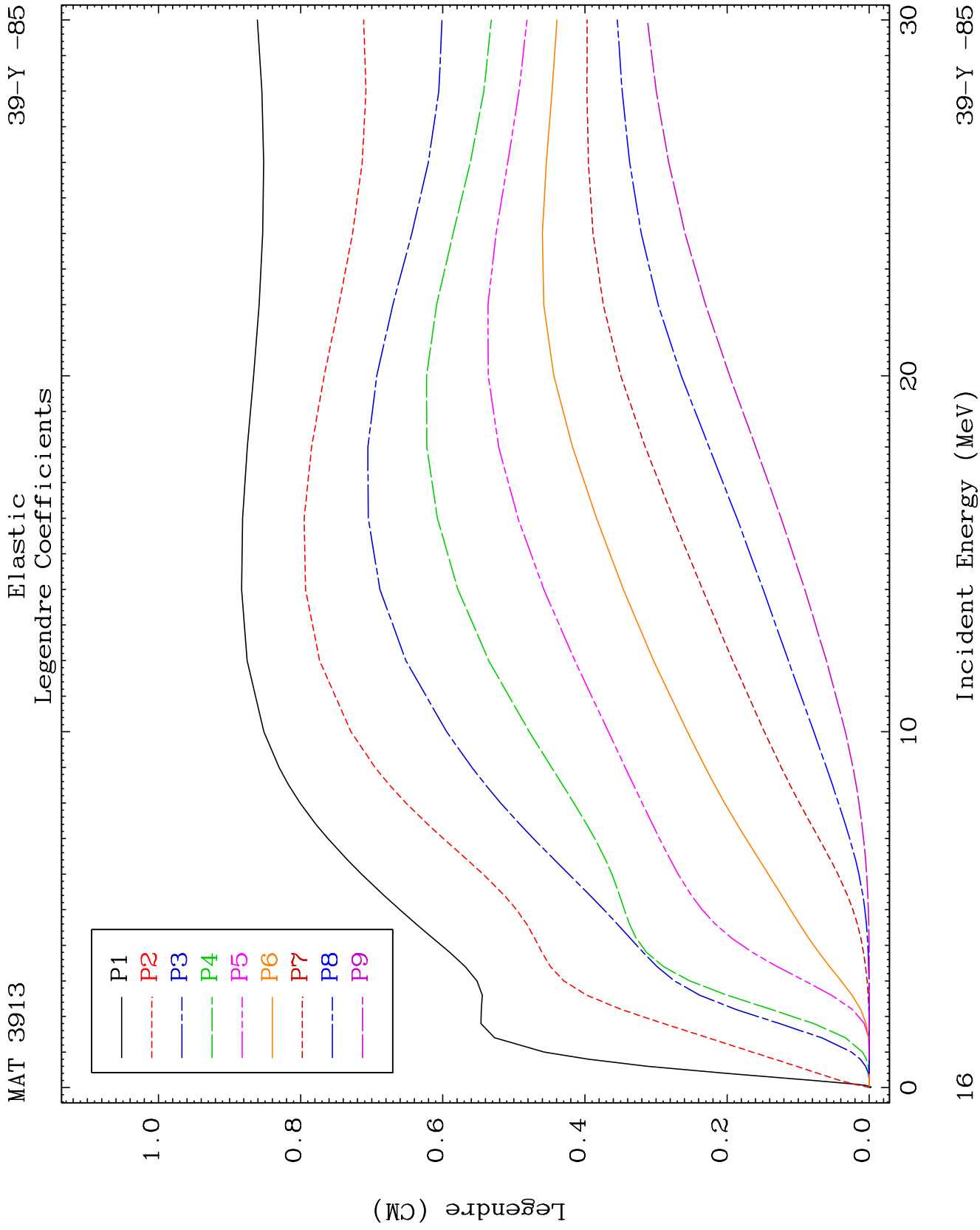
Incident Energy (MeV)

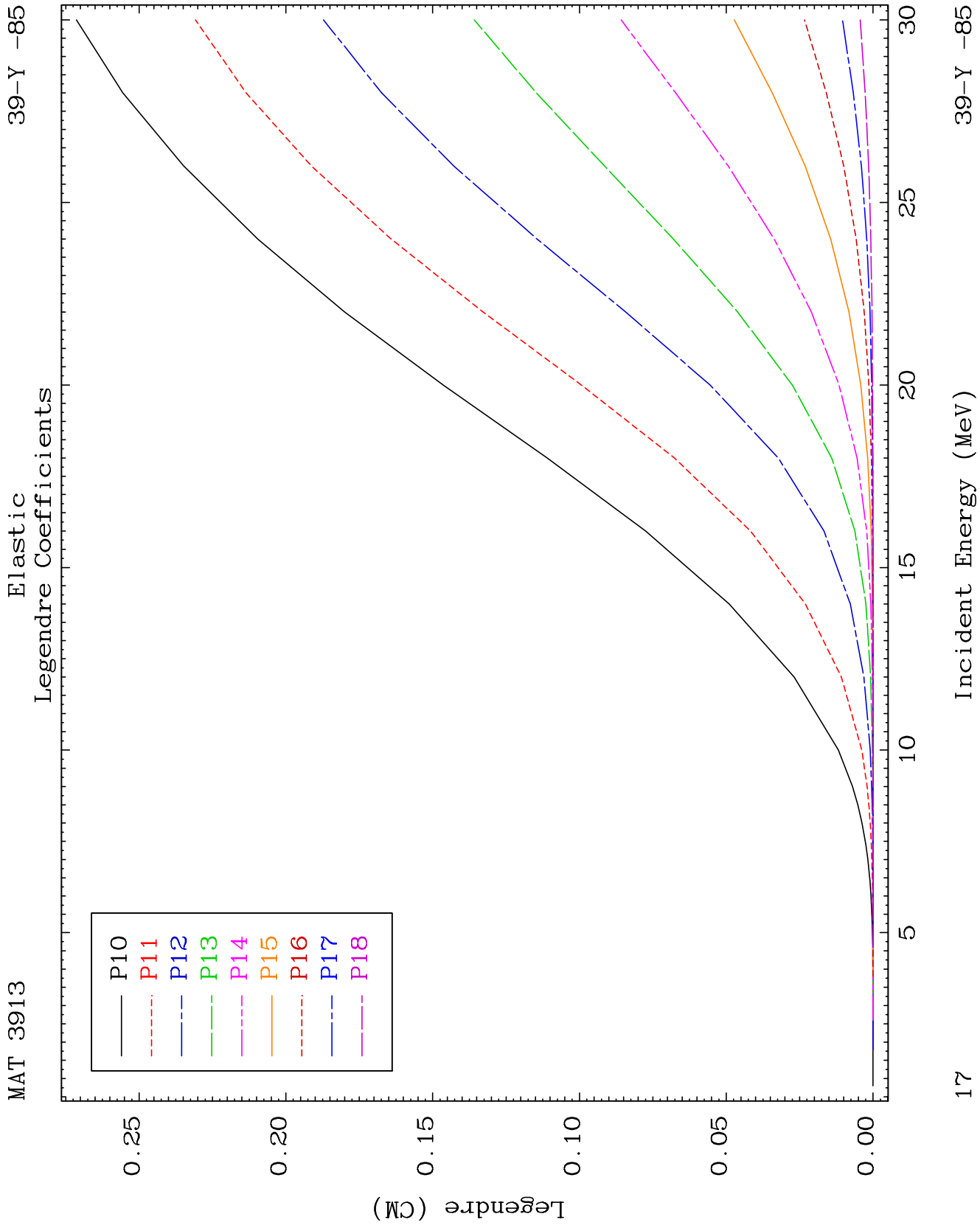
39-Y -85

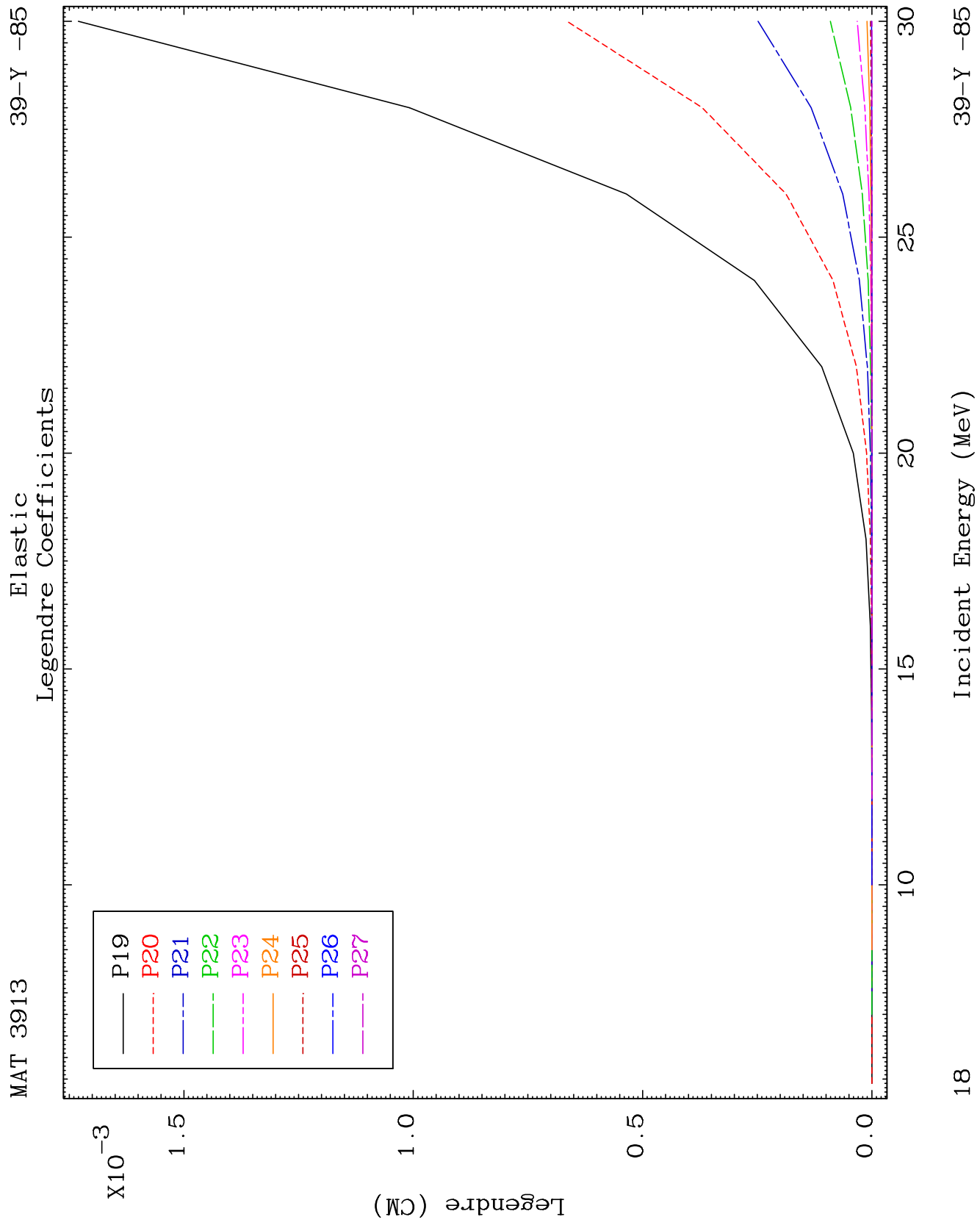


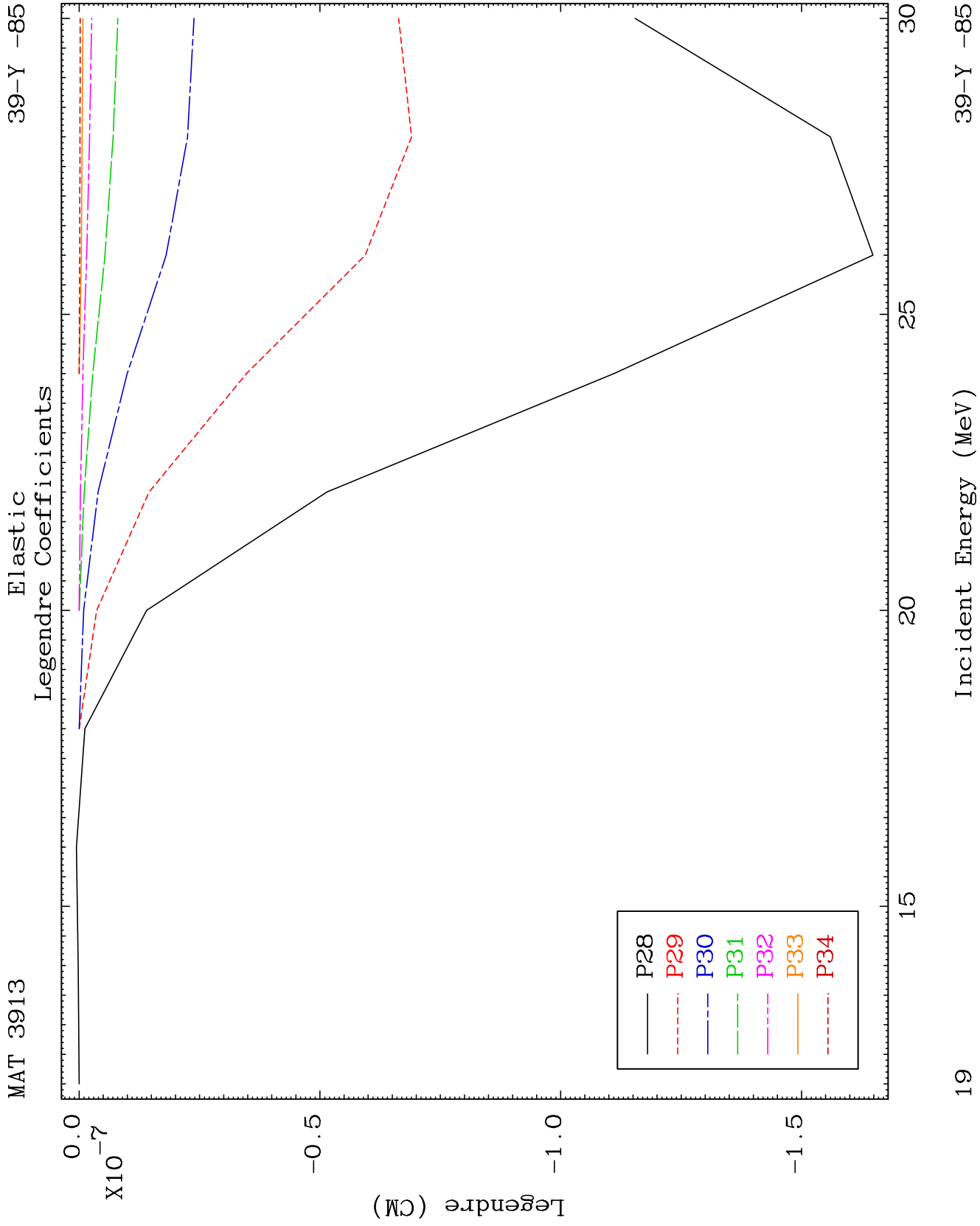


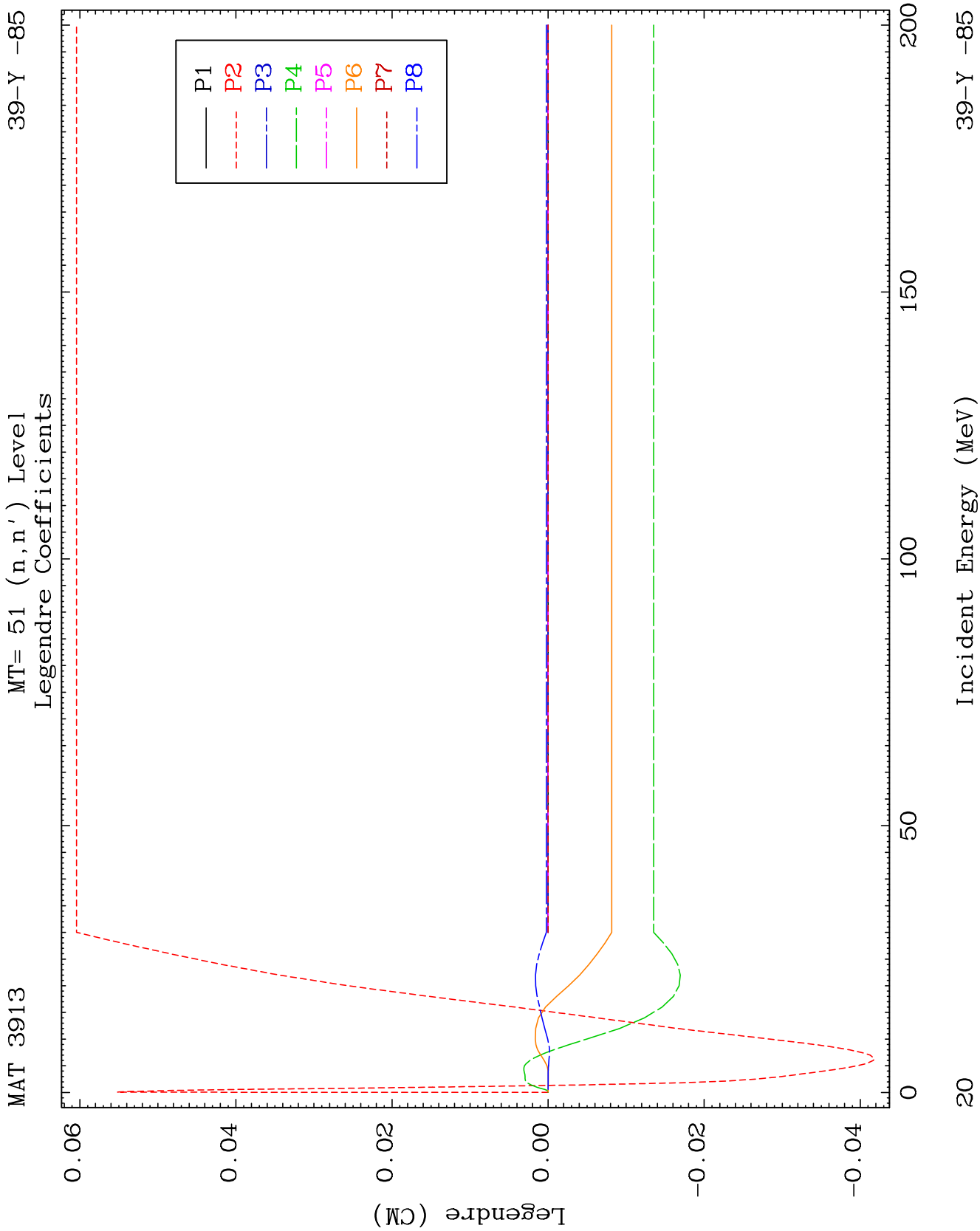








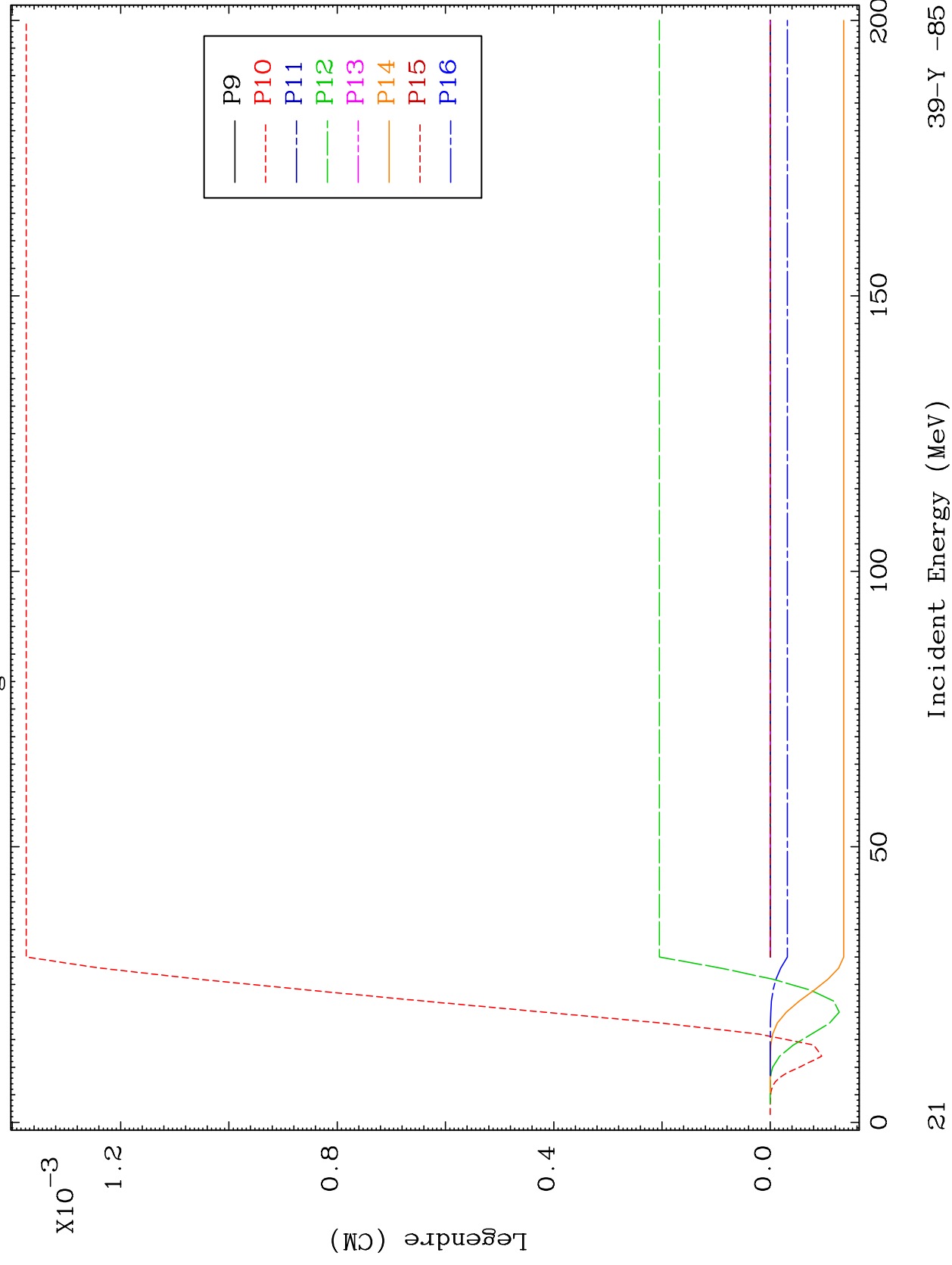


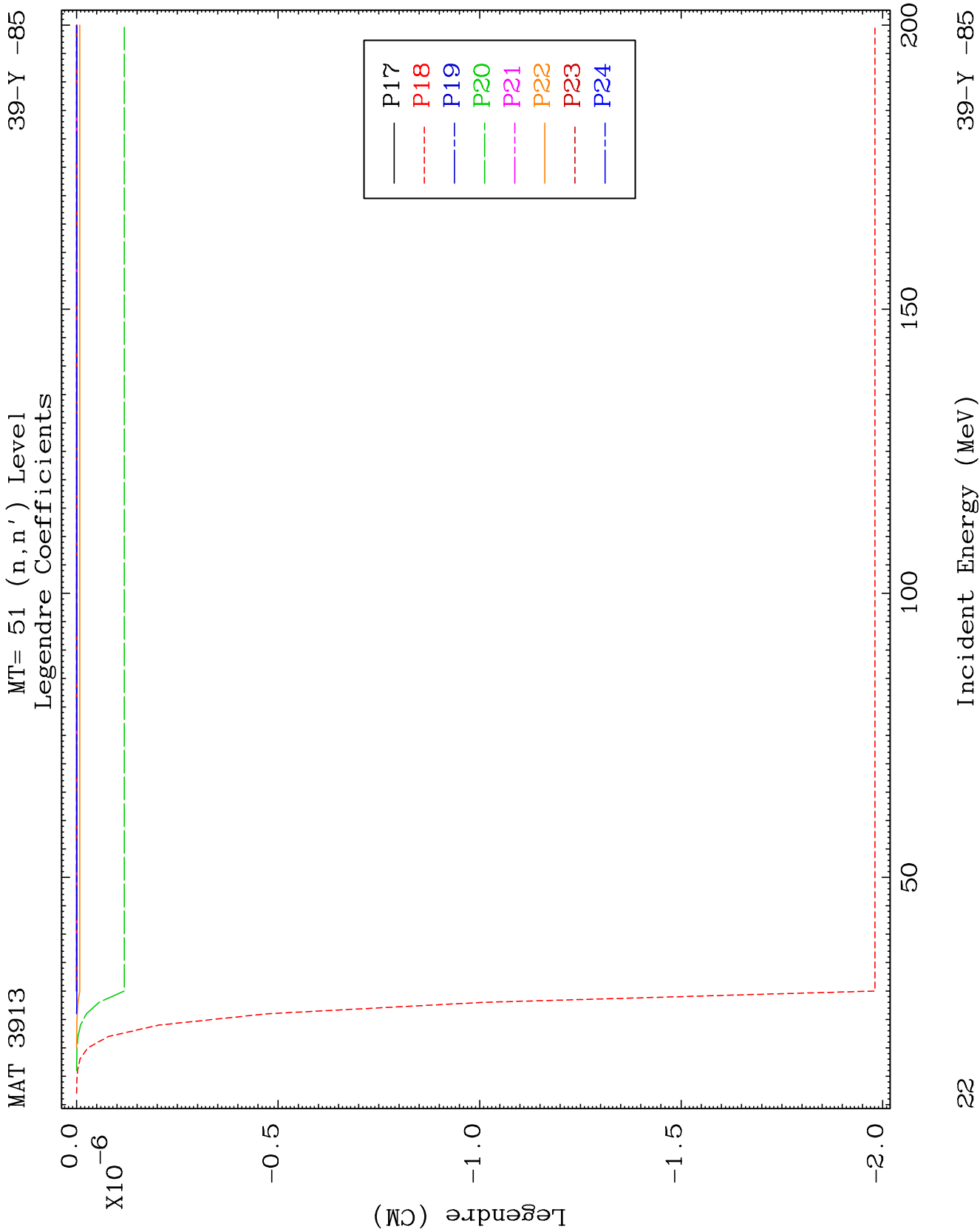


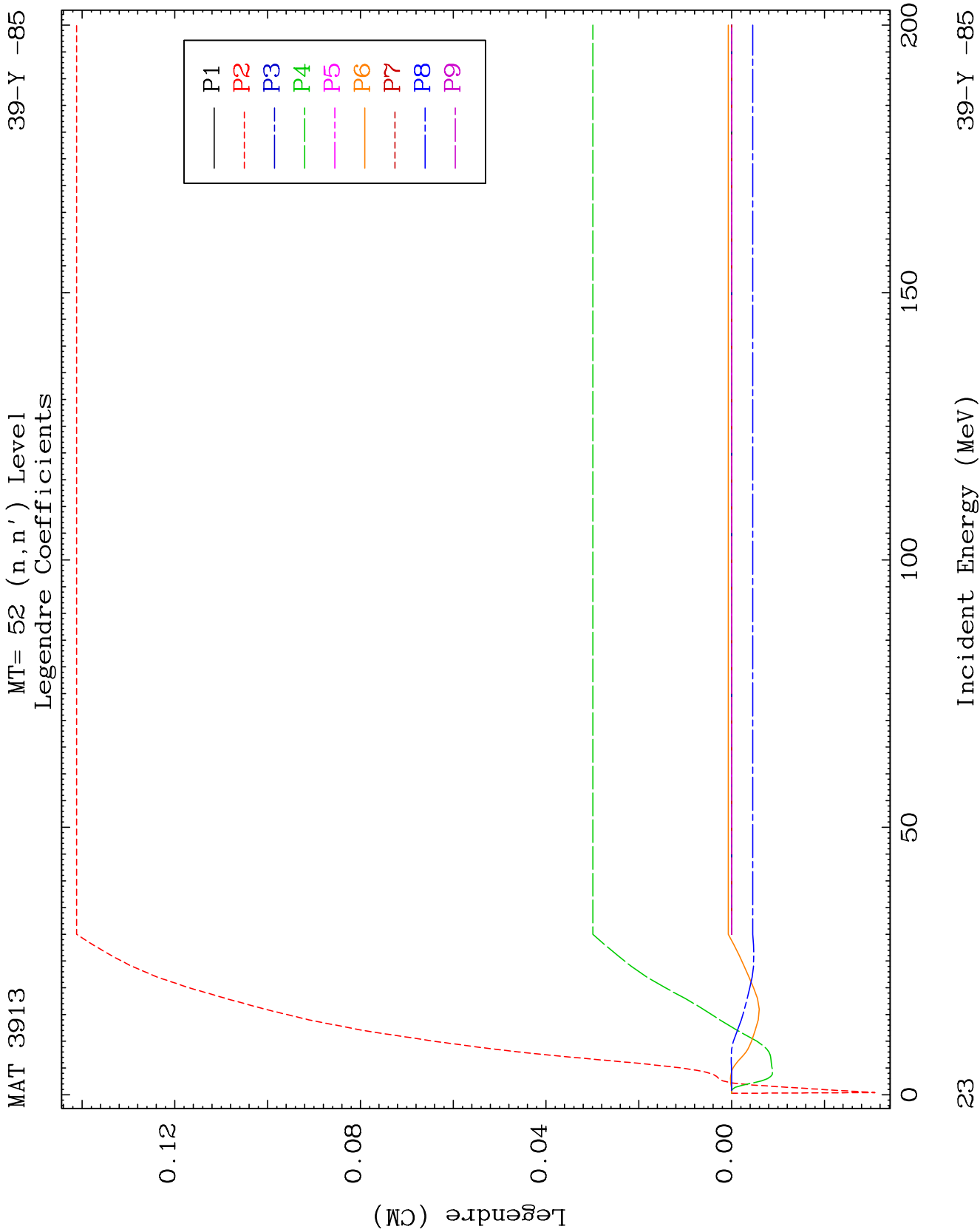
MAT 3913

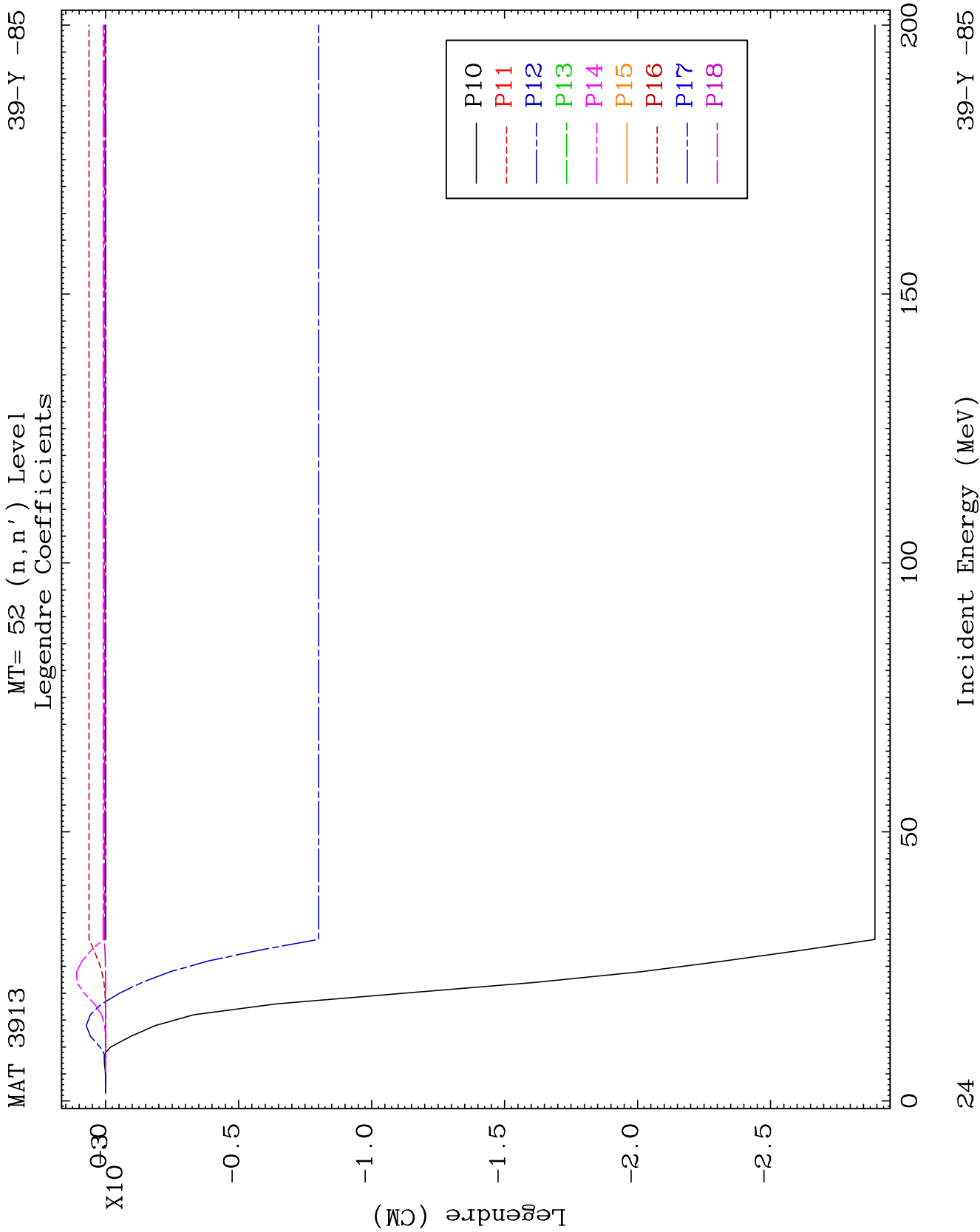
MT= 51 (n, n') Level
Legendre Coefficients

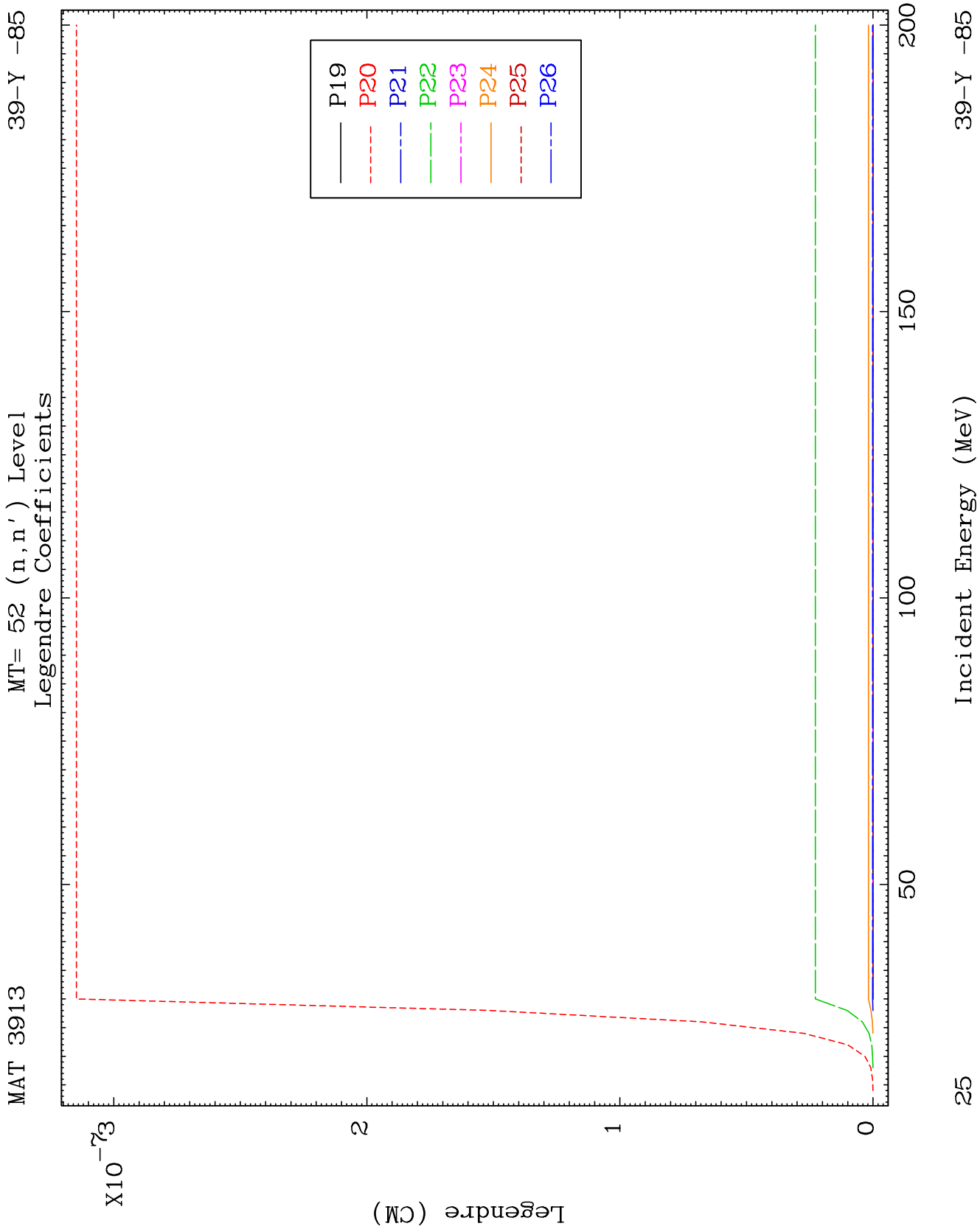
39-Y -85

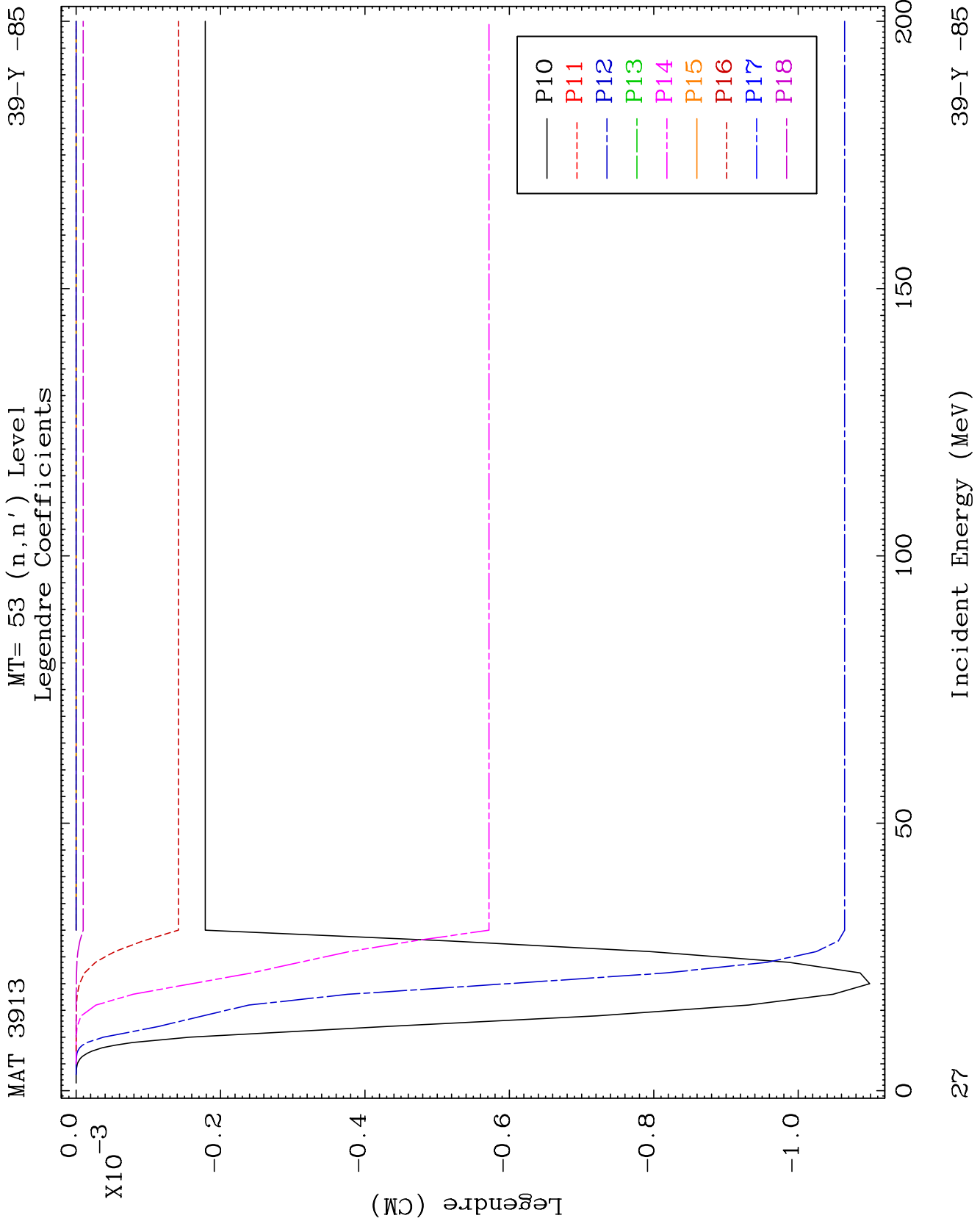


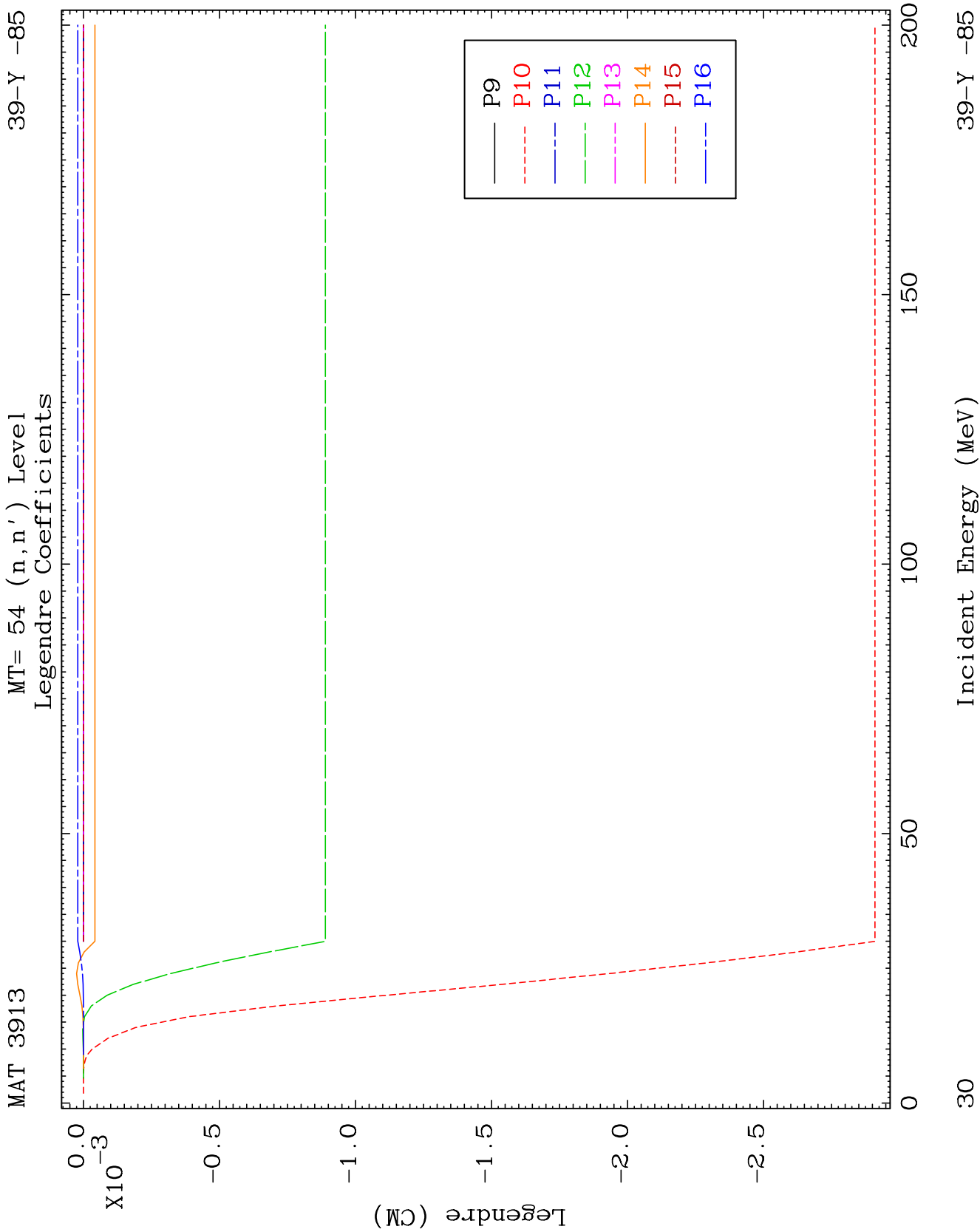








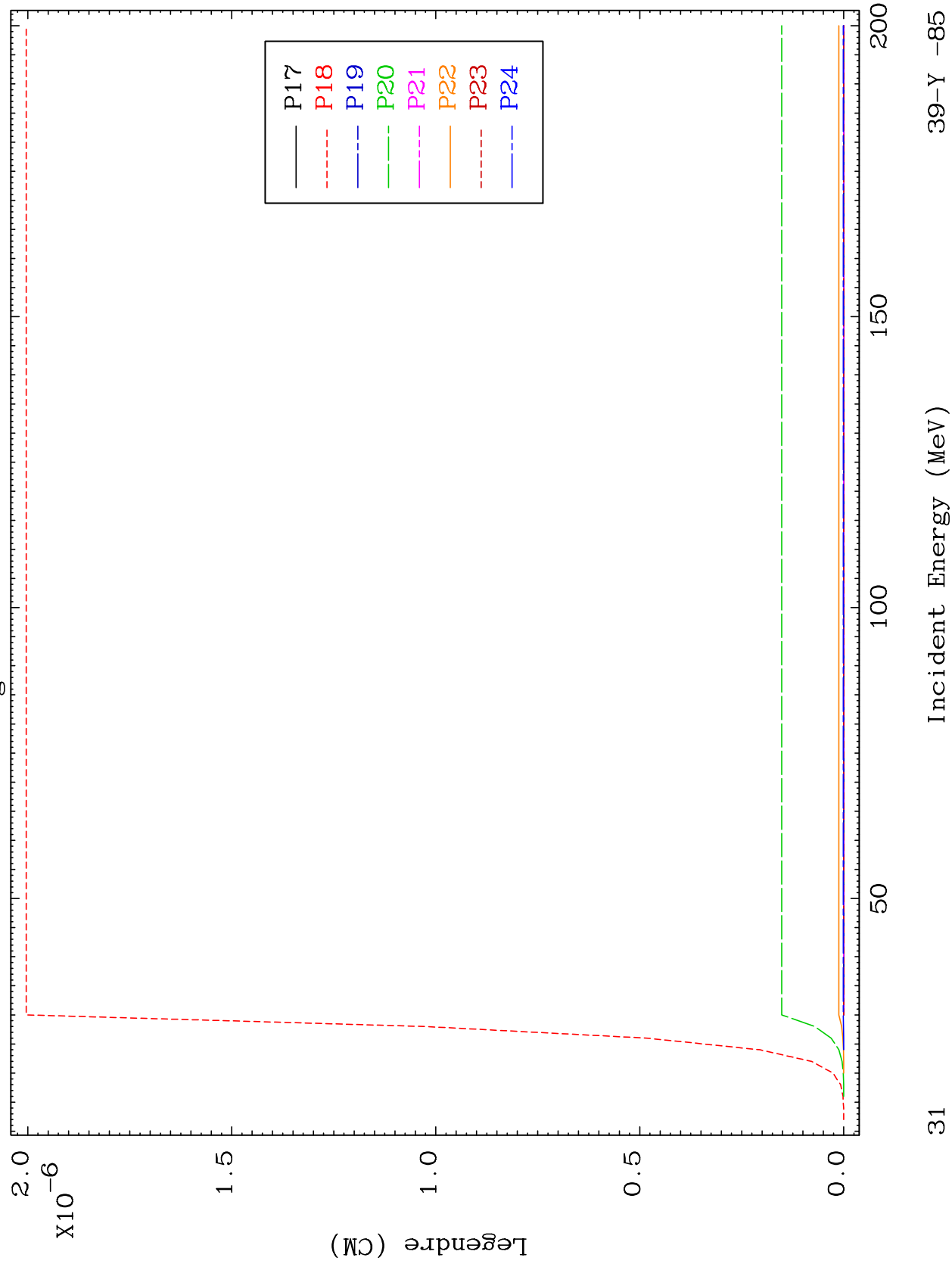


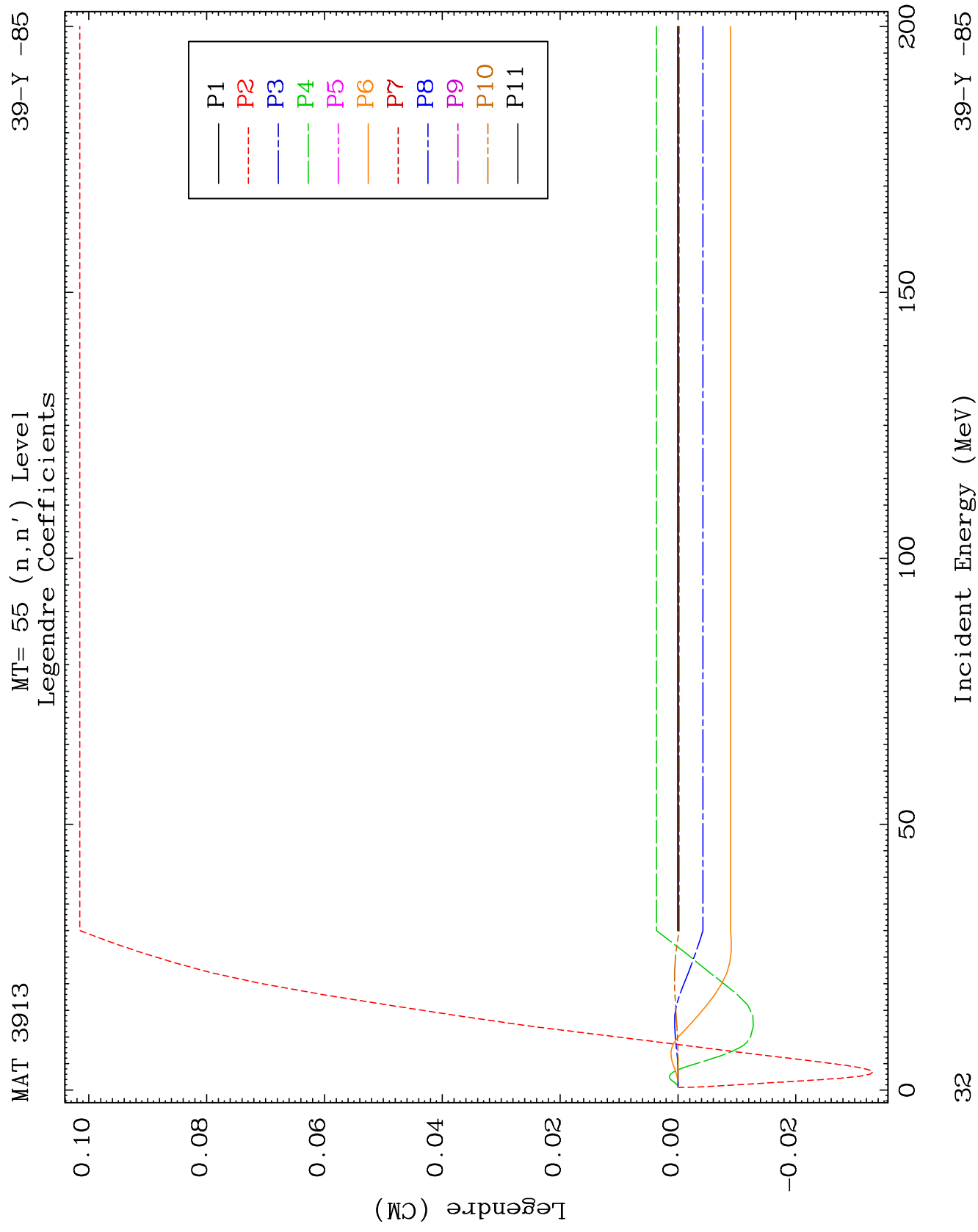


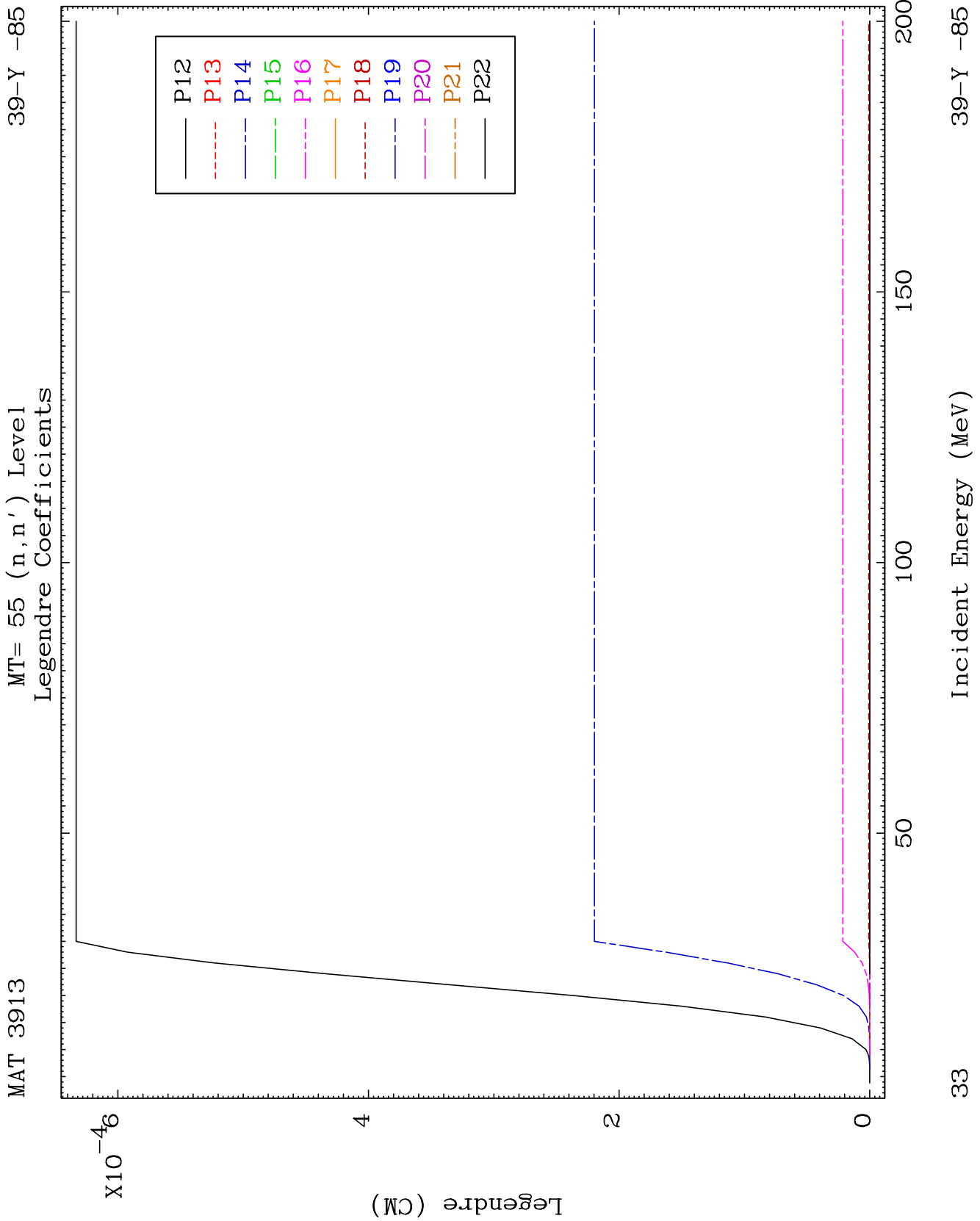
MAT 3913

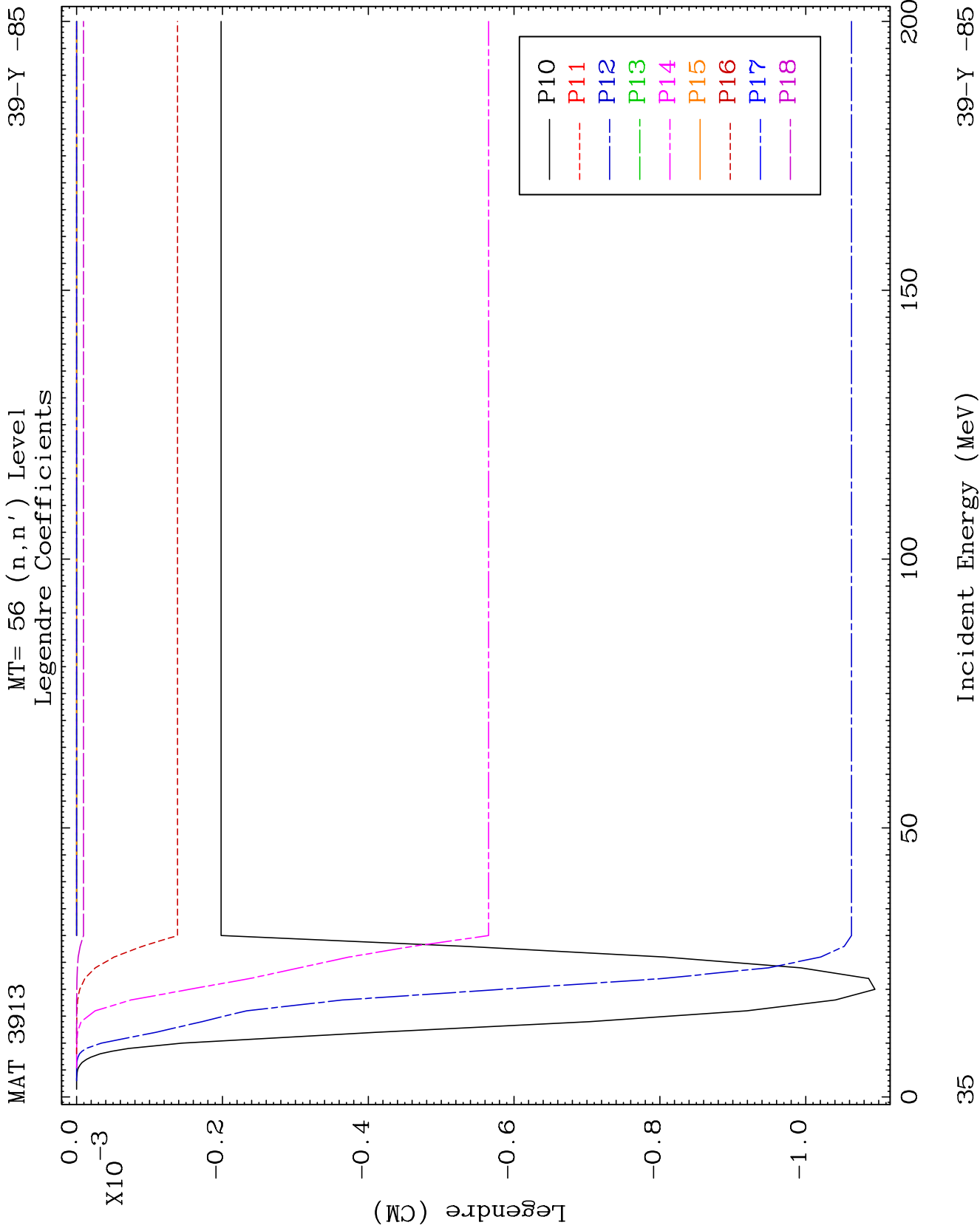
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Legendre Coefficients

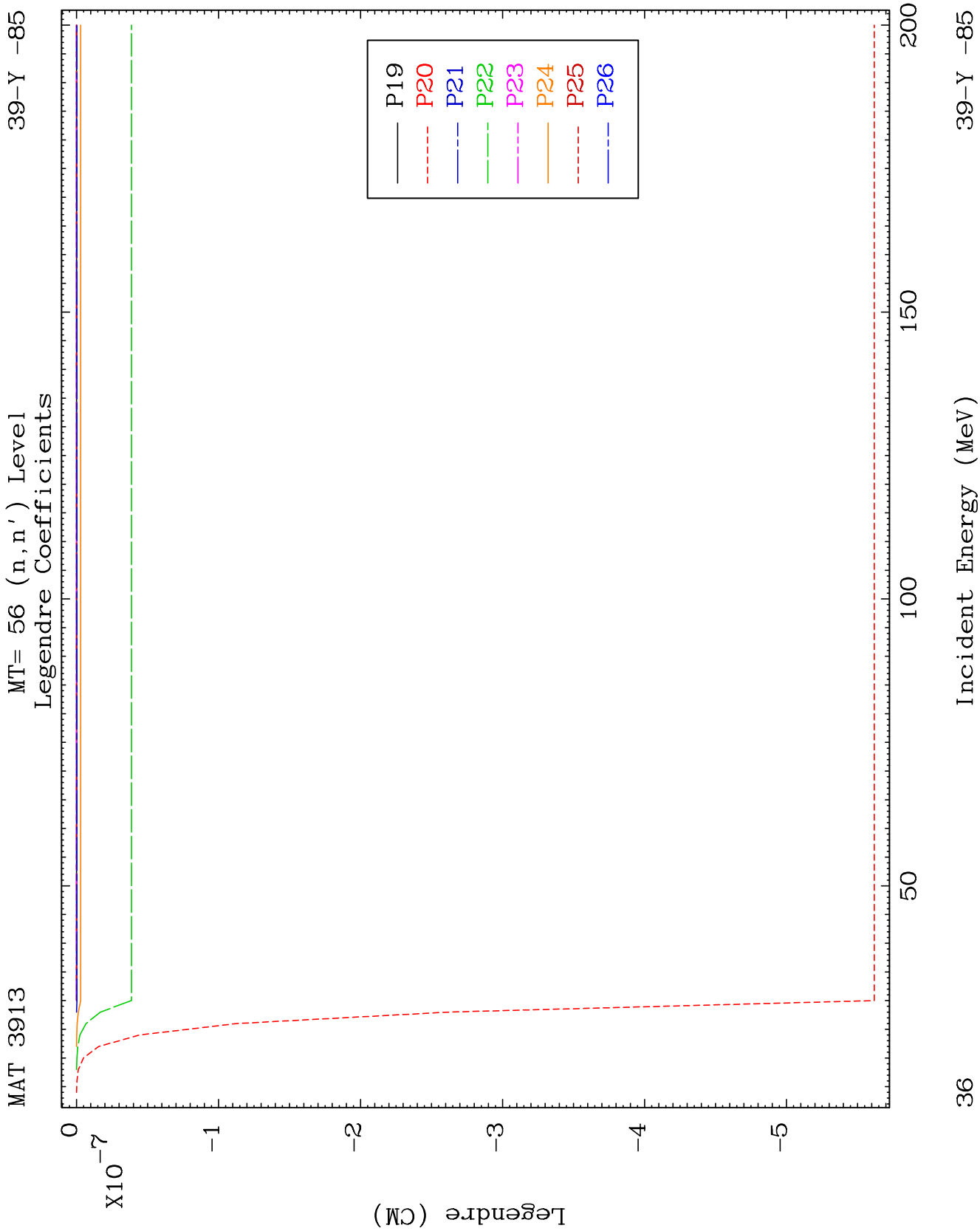
39-Y -85







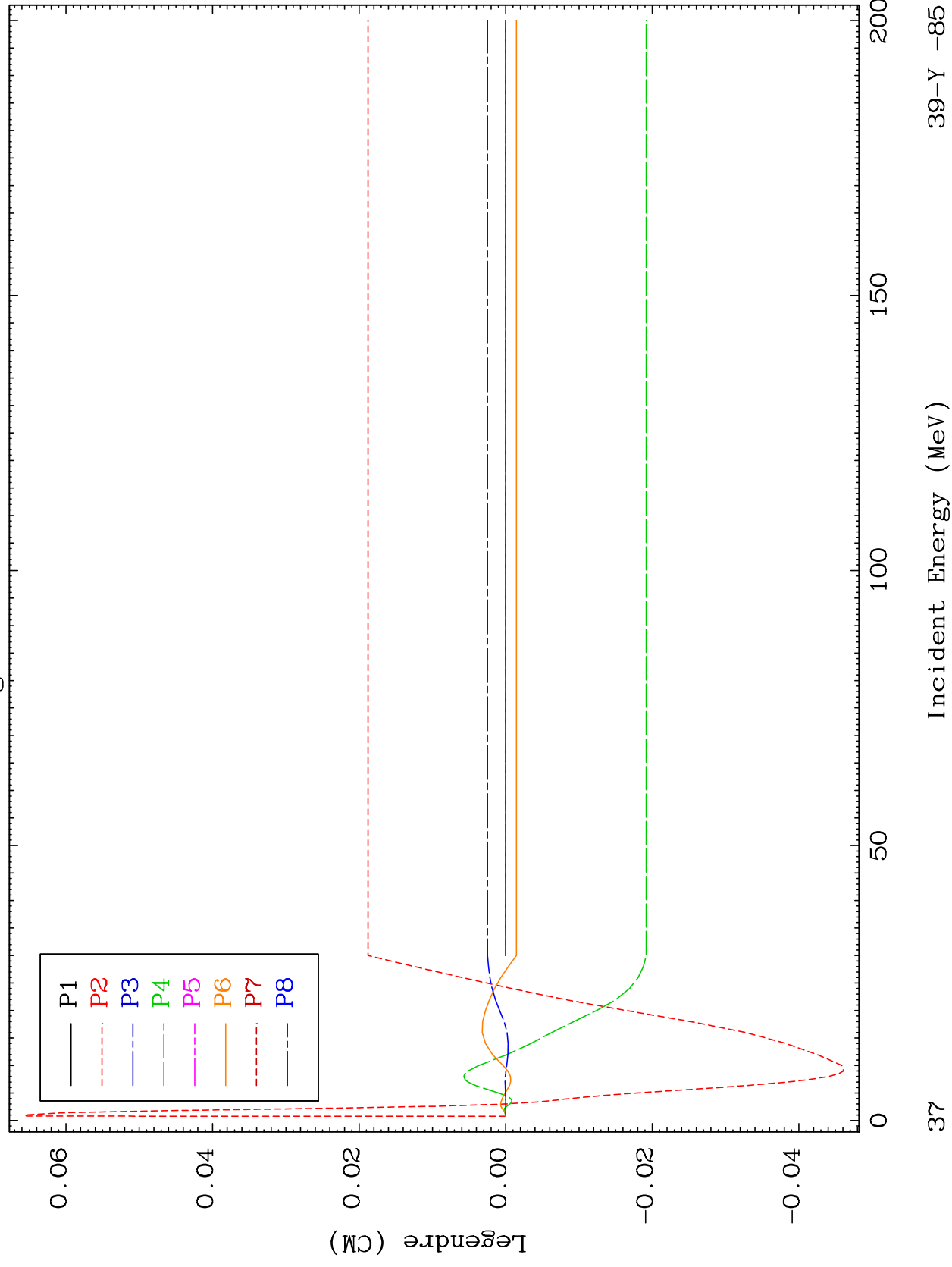


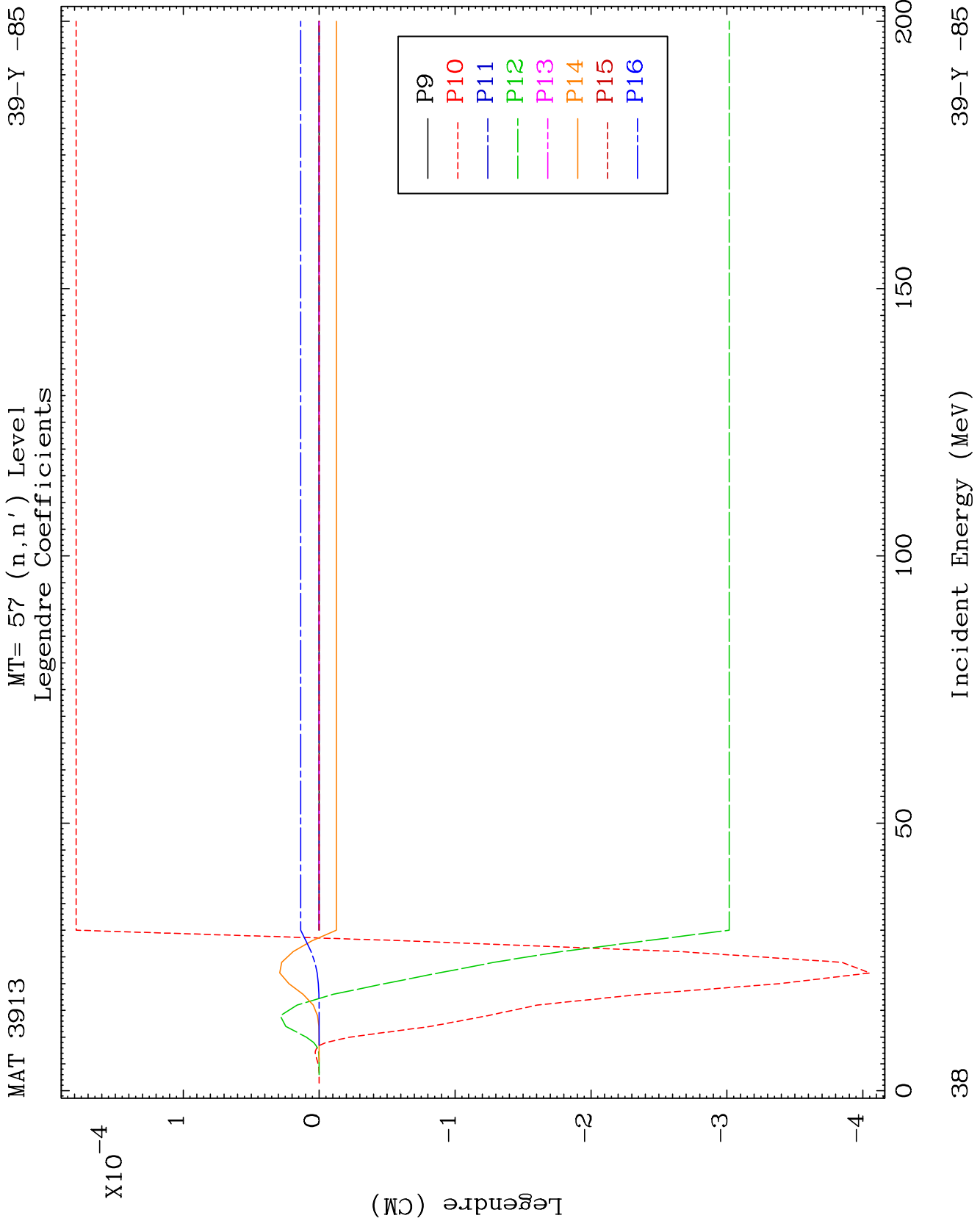


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MT= 57 (n, n') Level
Legendre Coefficients

39-Y -85

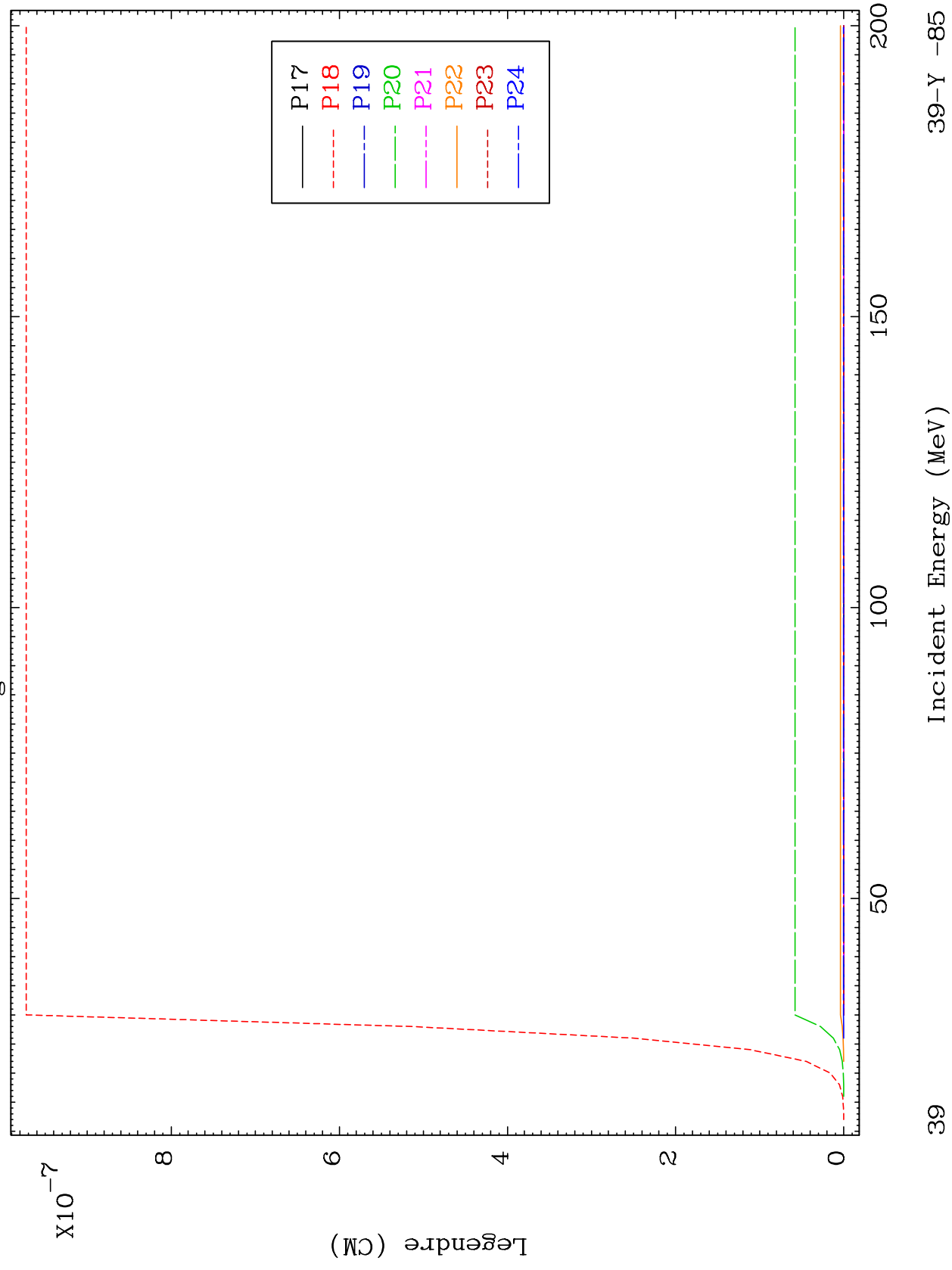


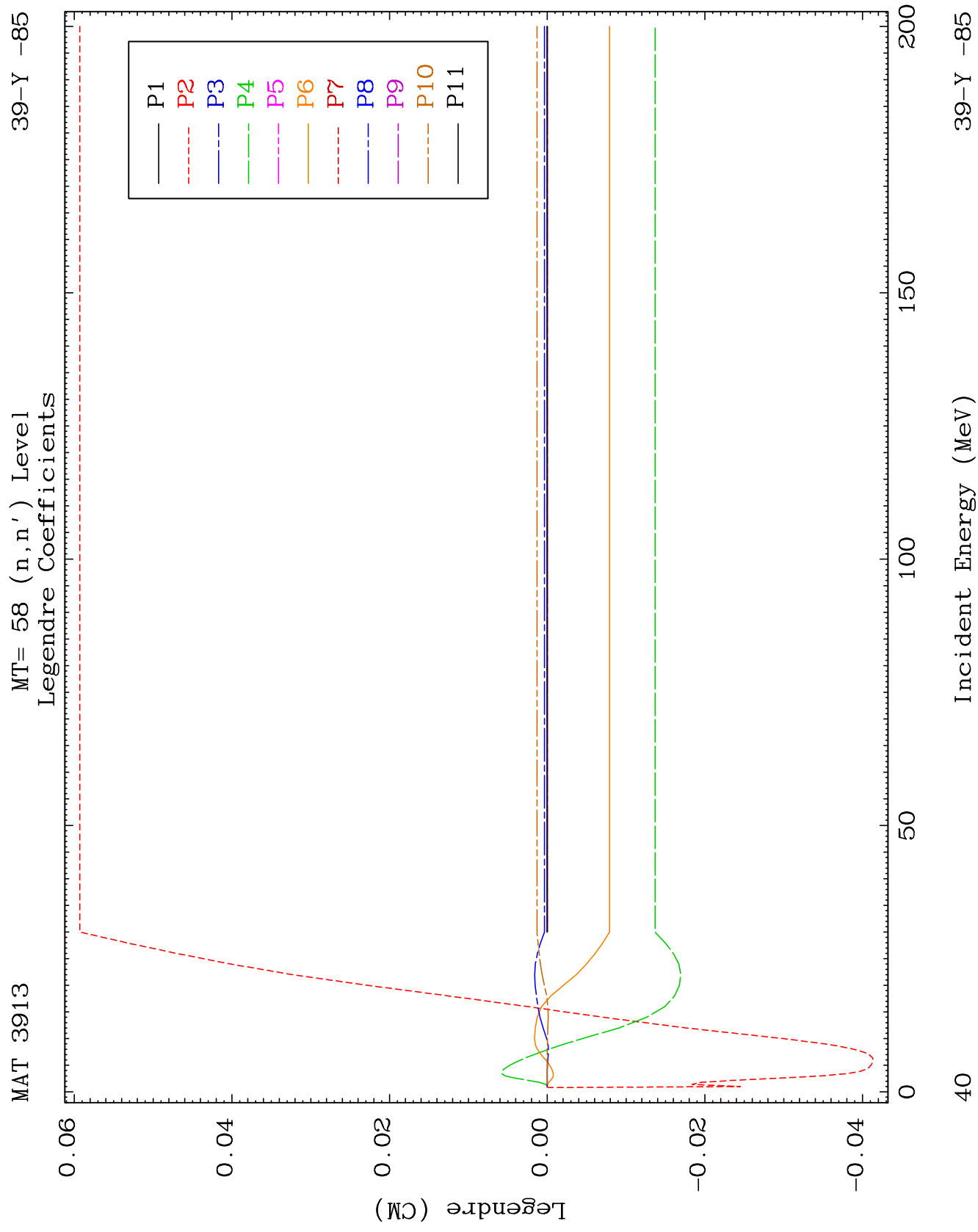


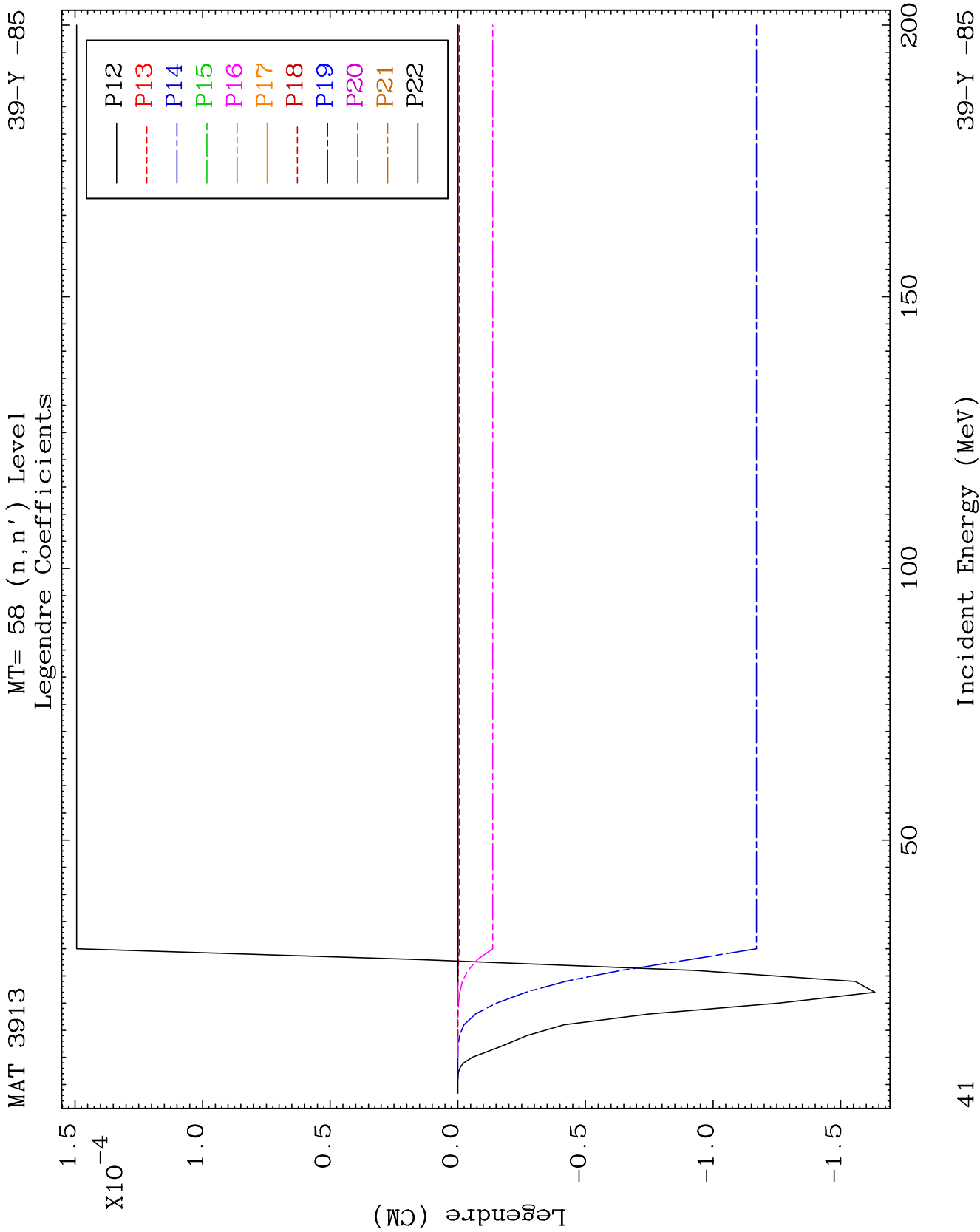
MAT 3913

MT= 57 (n, n') Level
Legendre Coefficients

39-Y -85



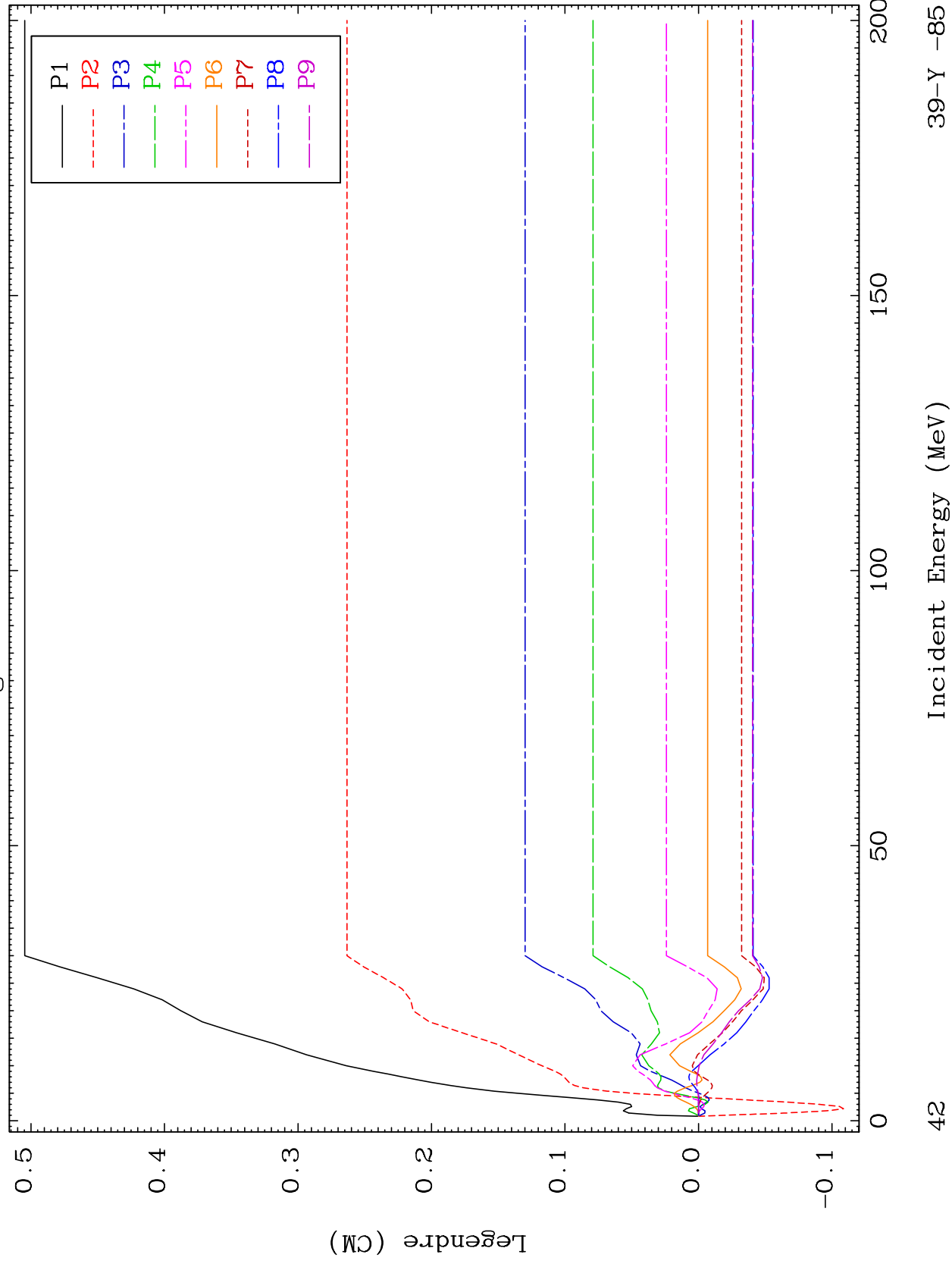


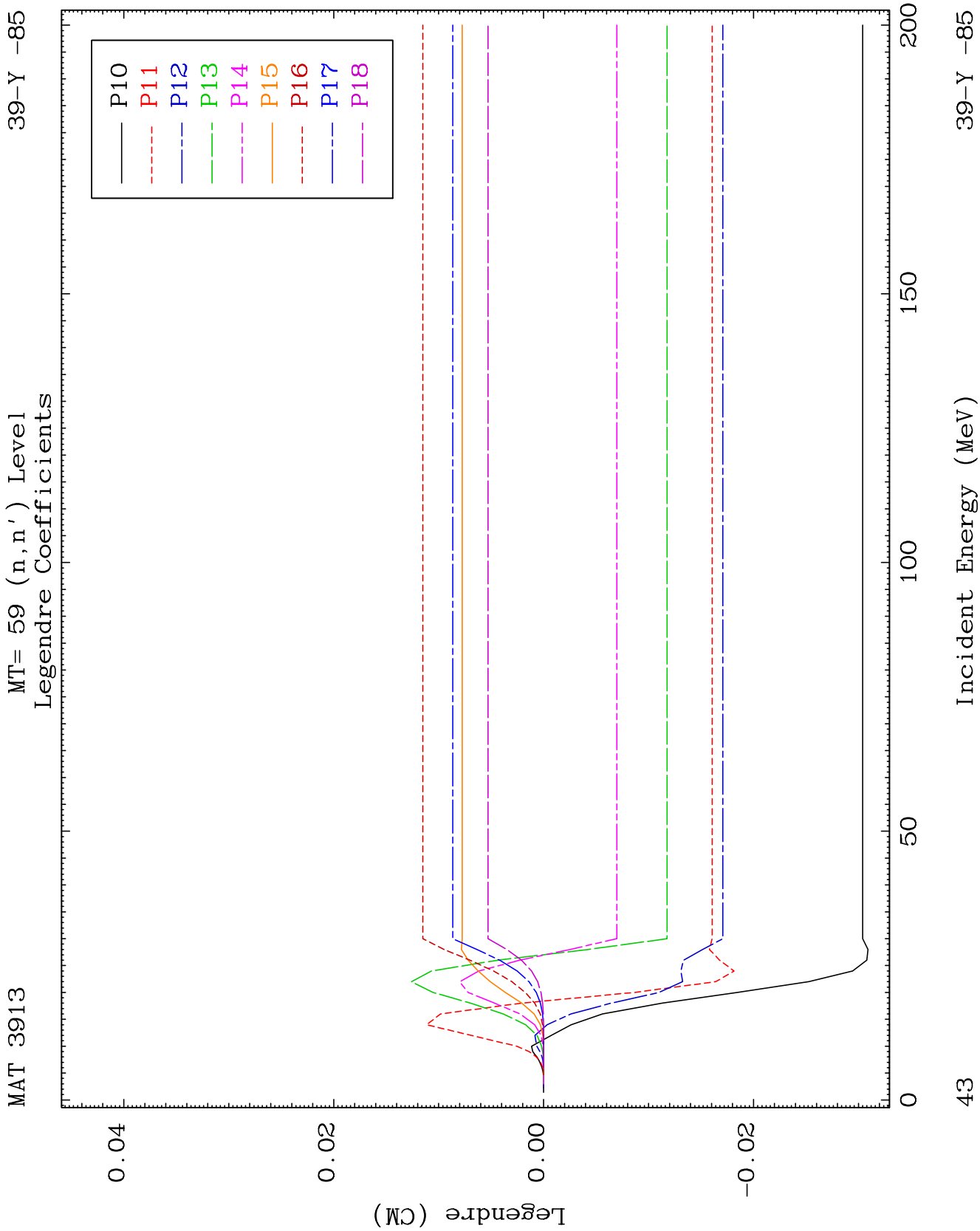


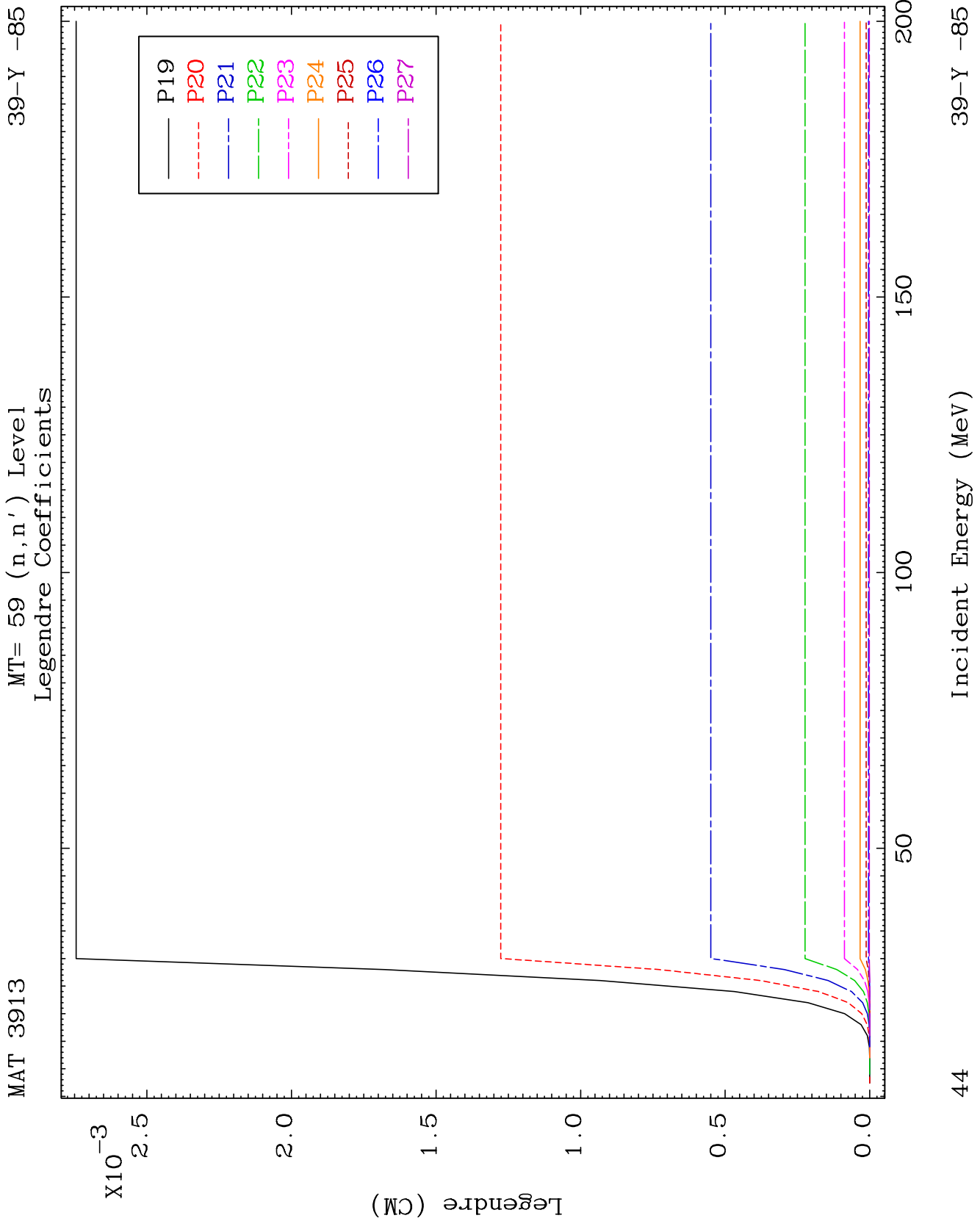
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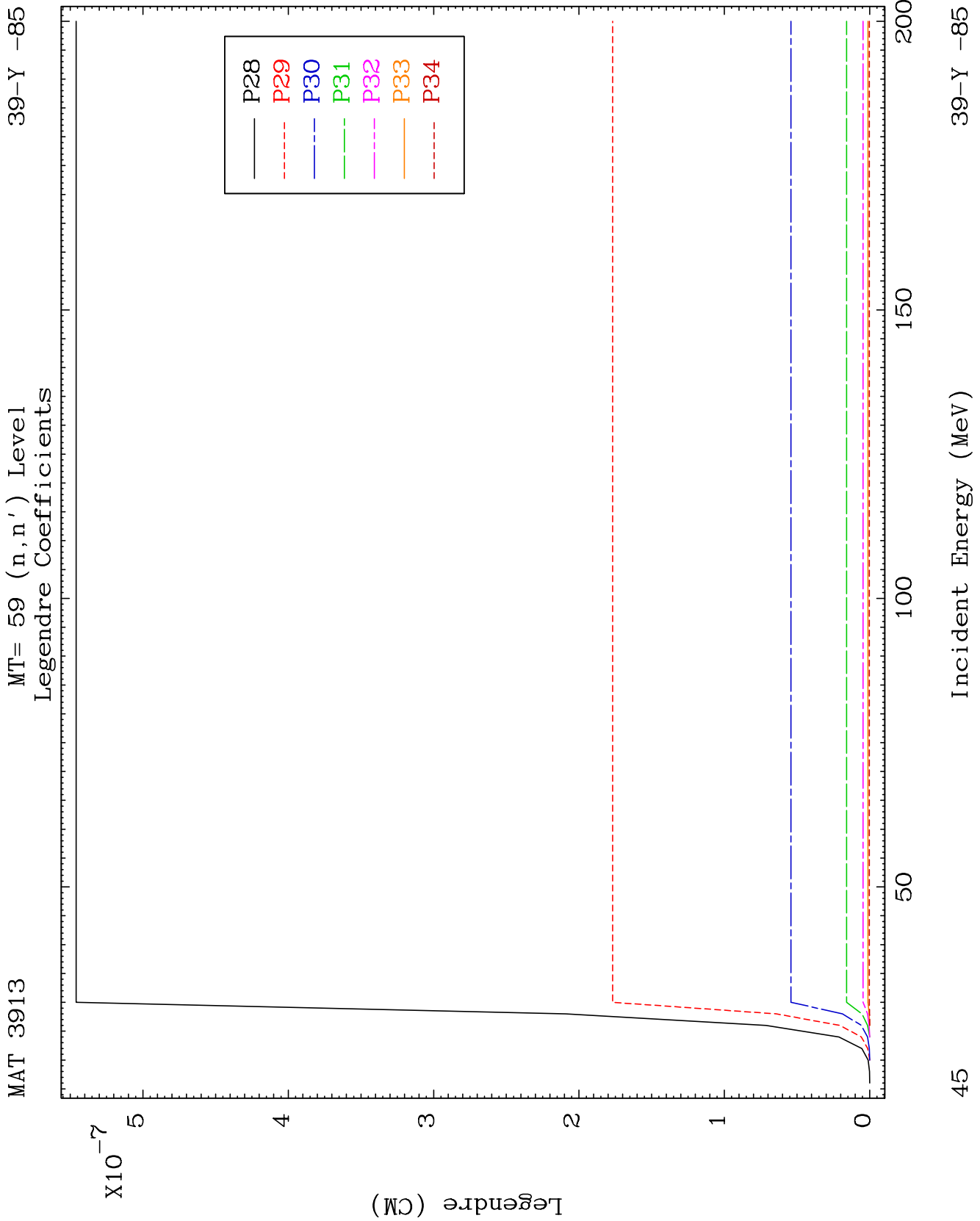
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Legendre Coefficients

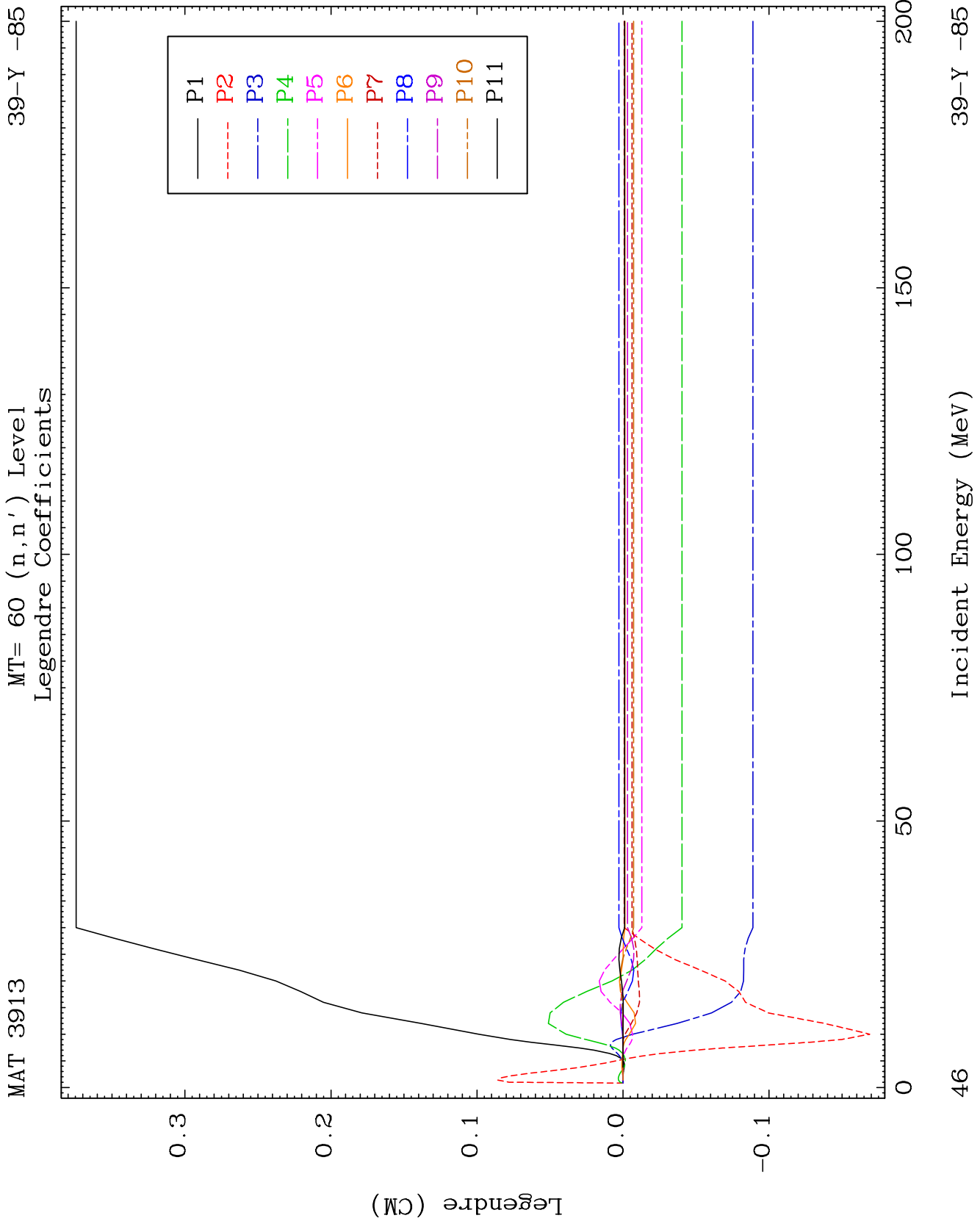
39-Y -85

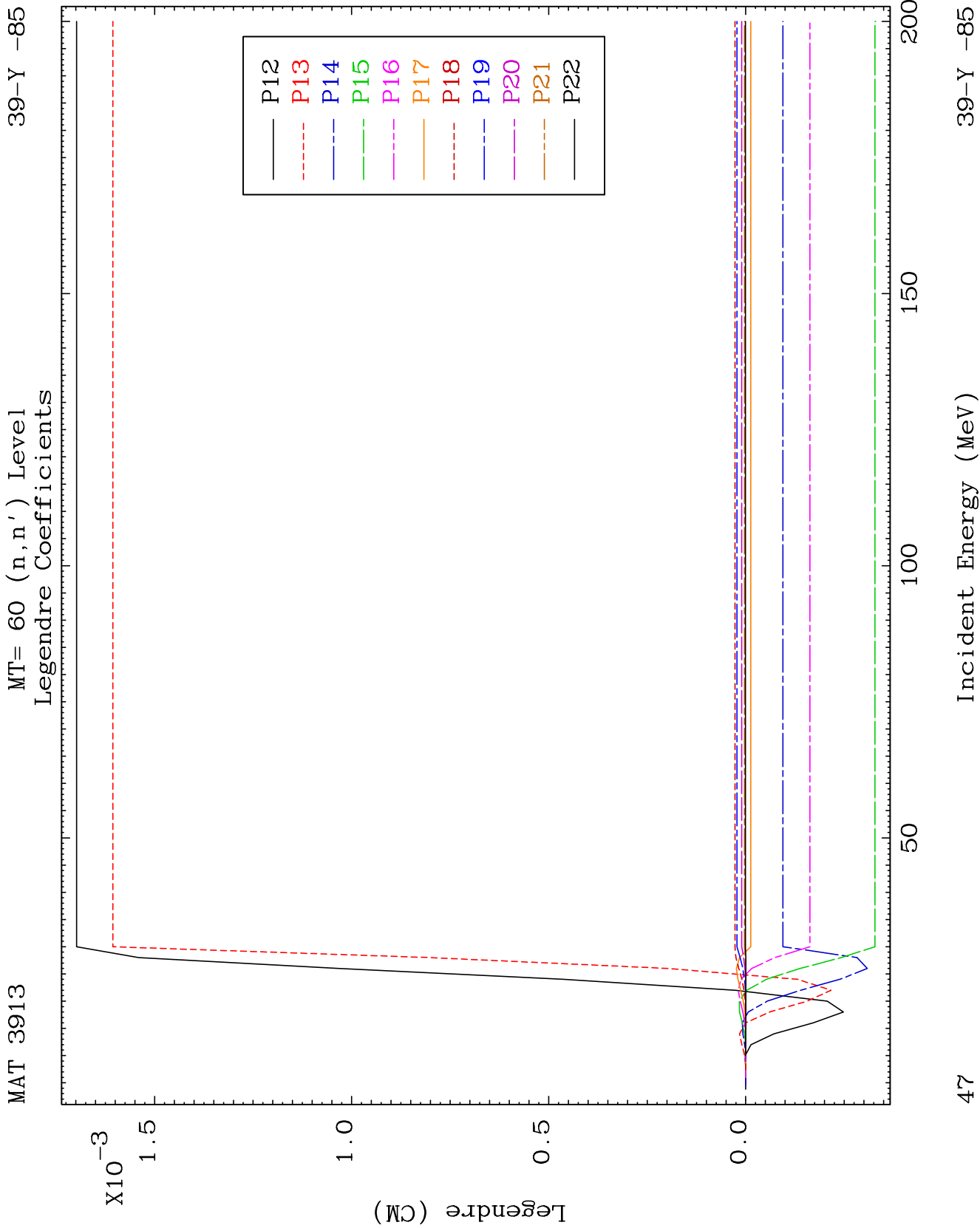


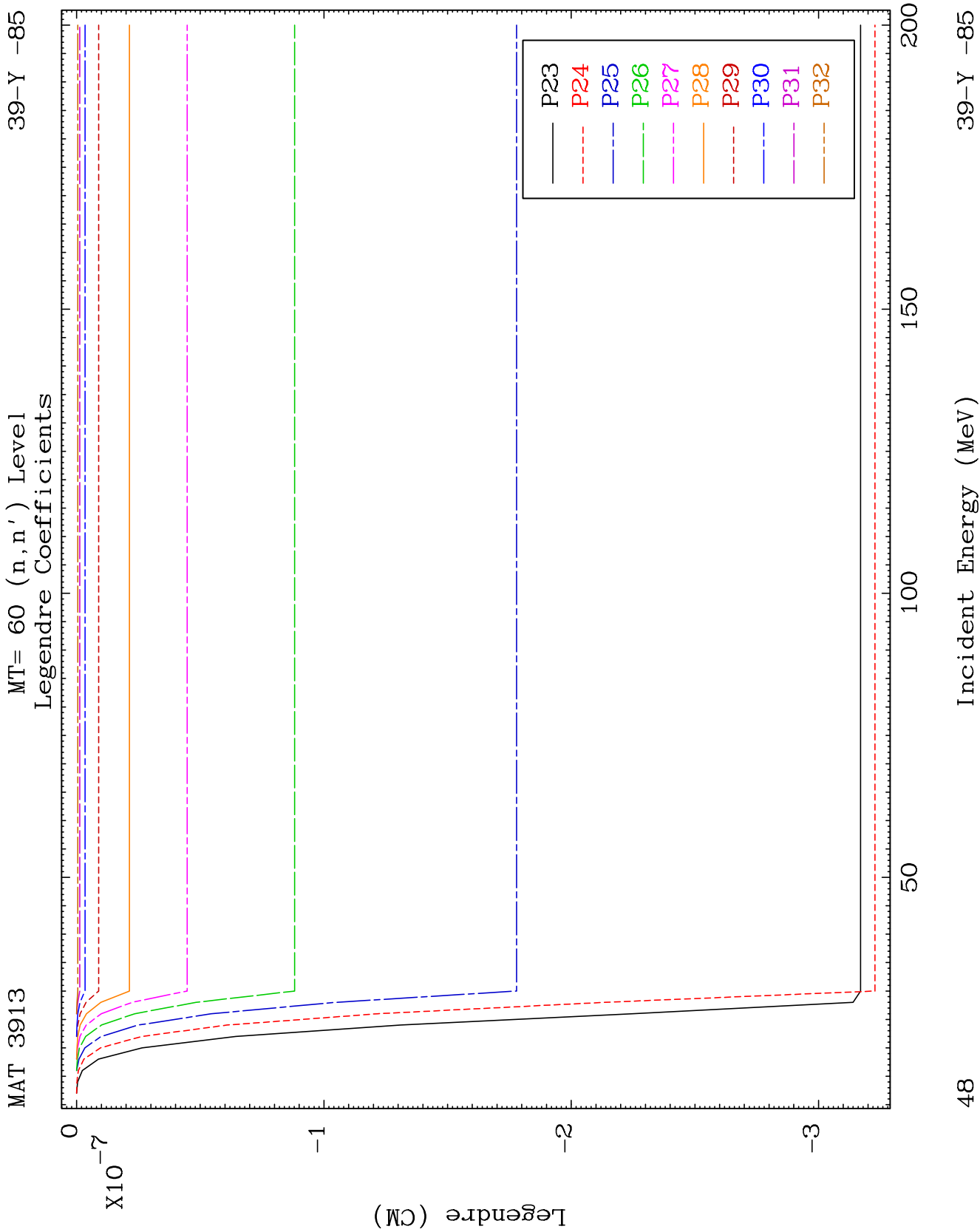








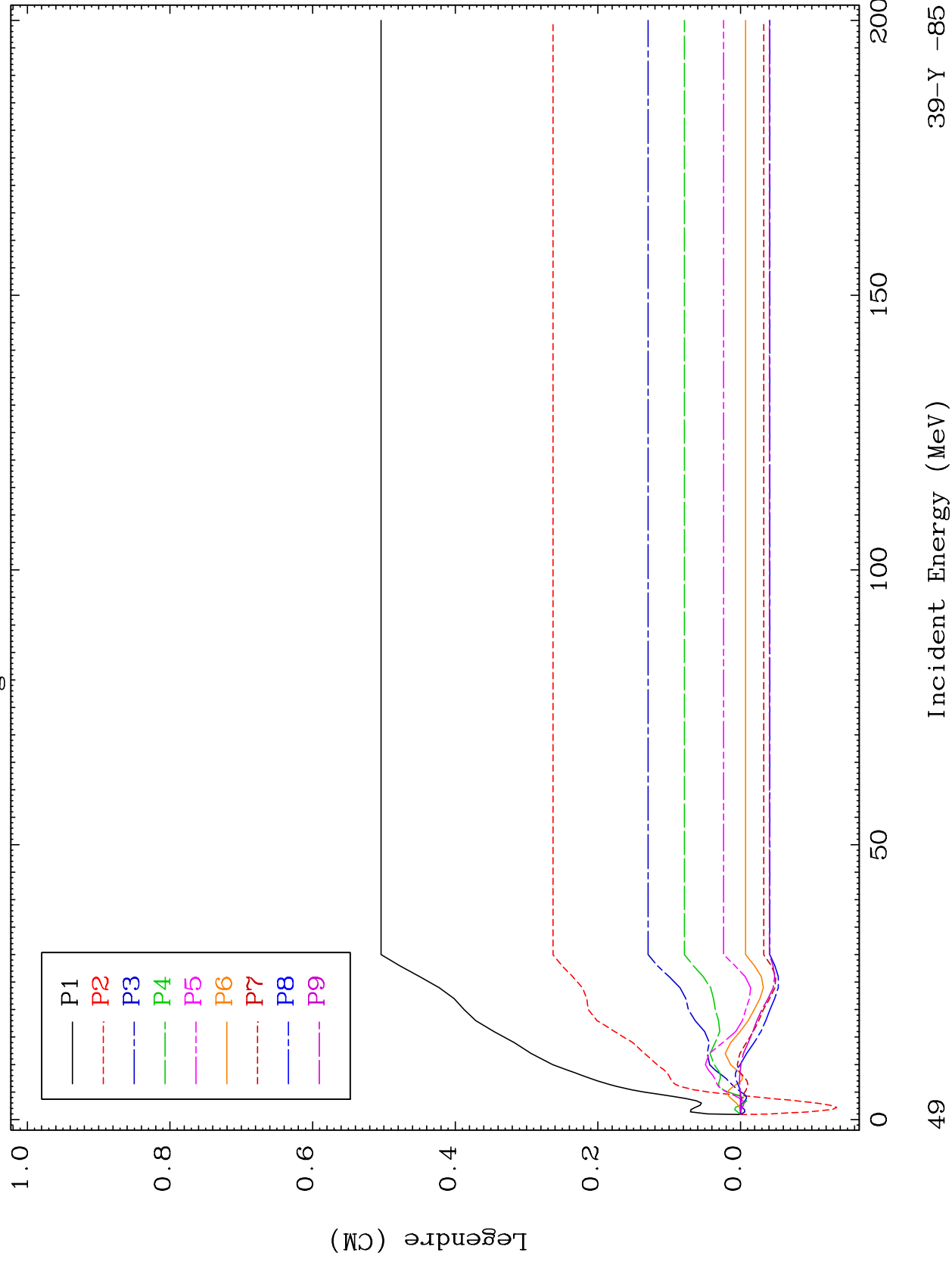


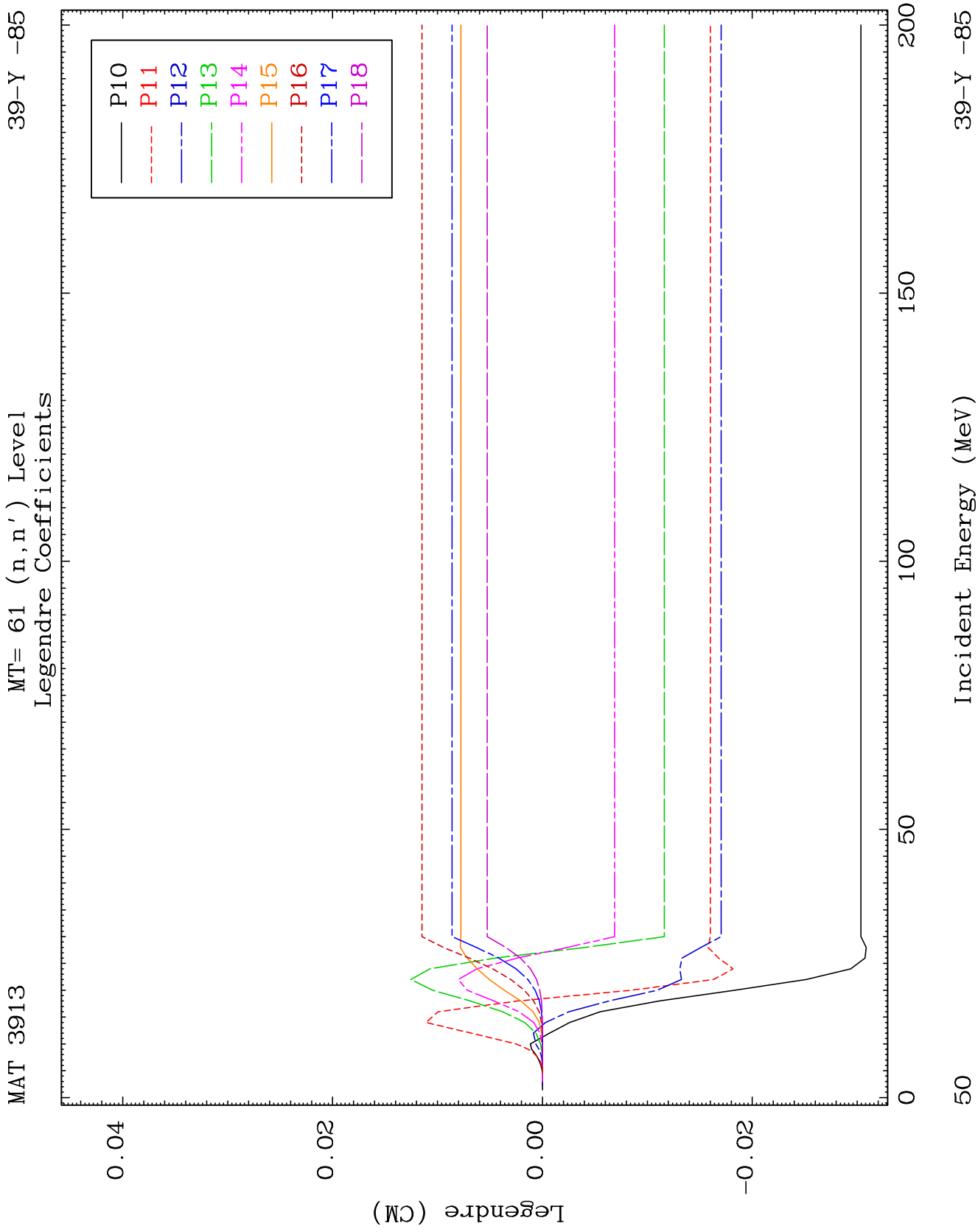


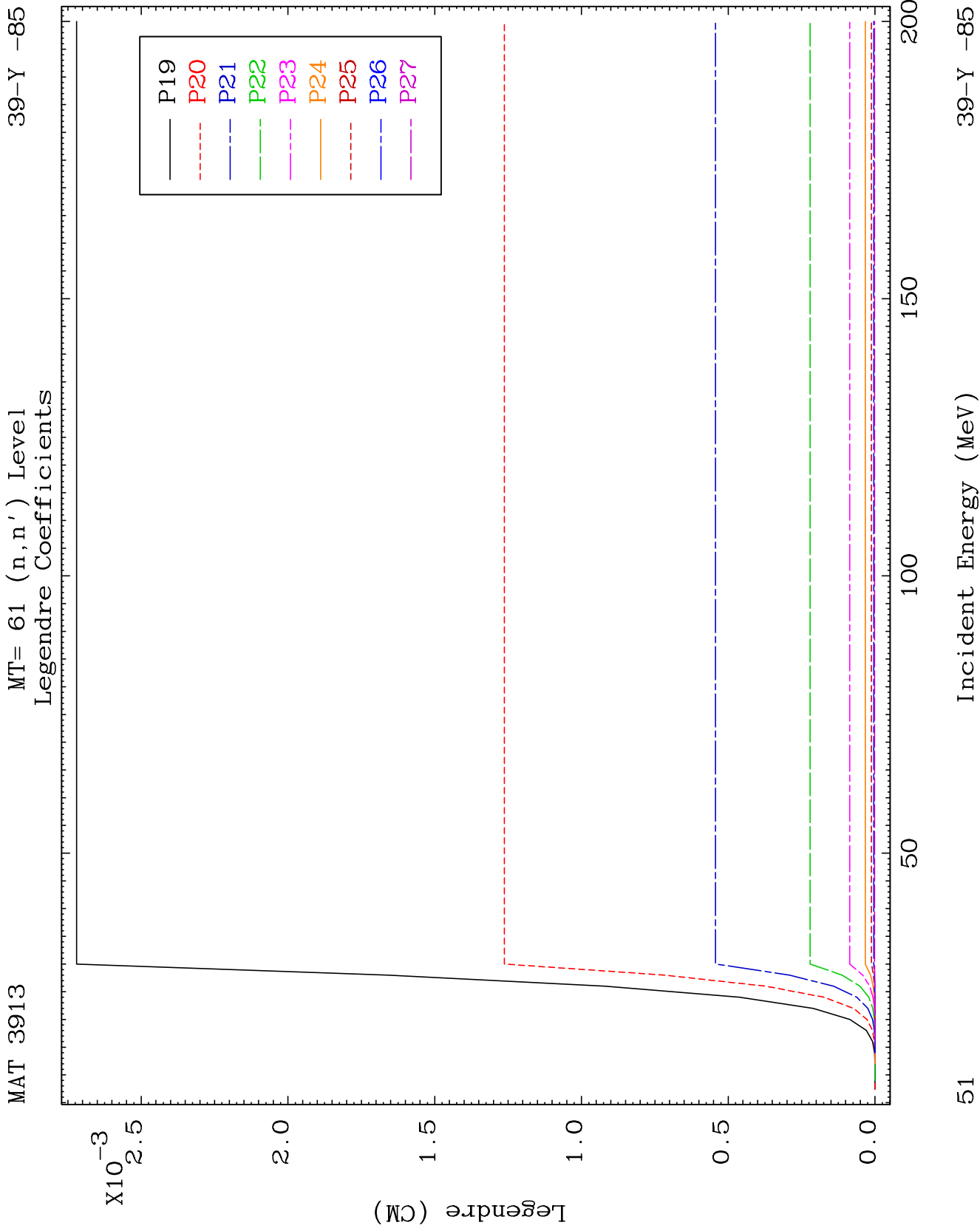
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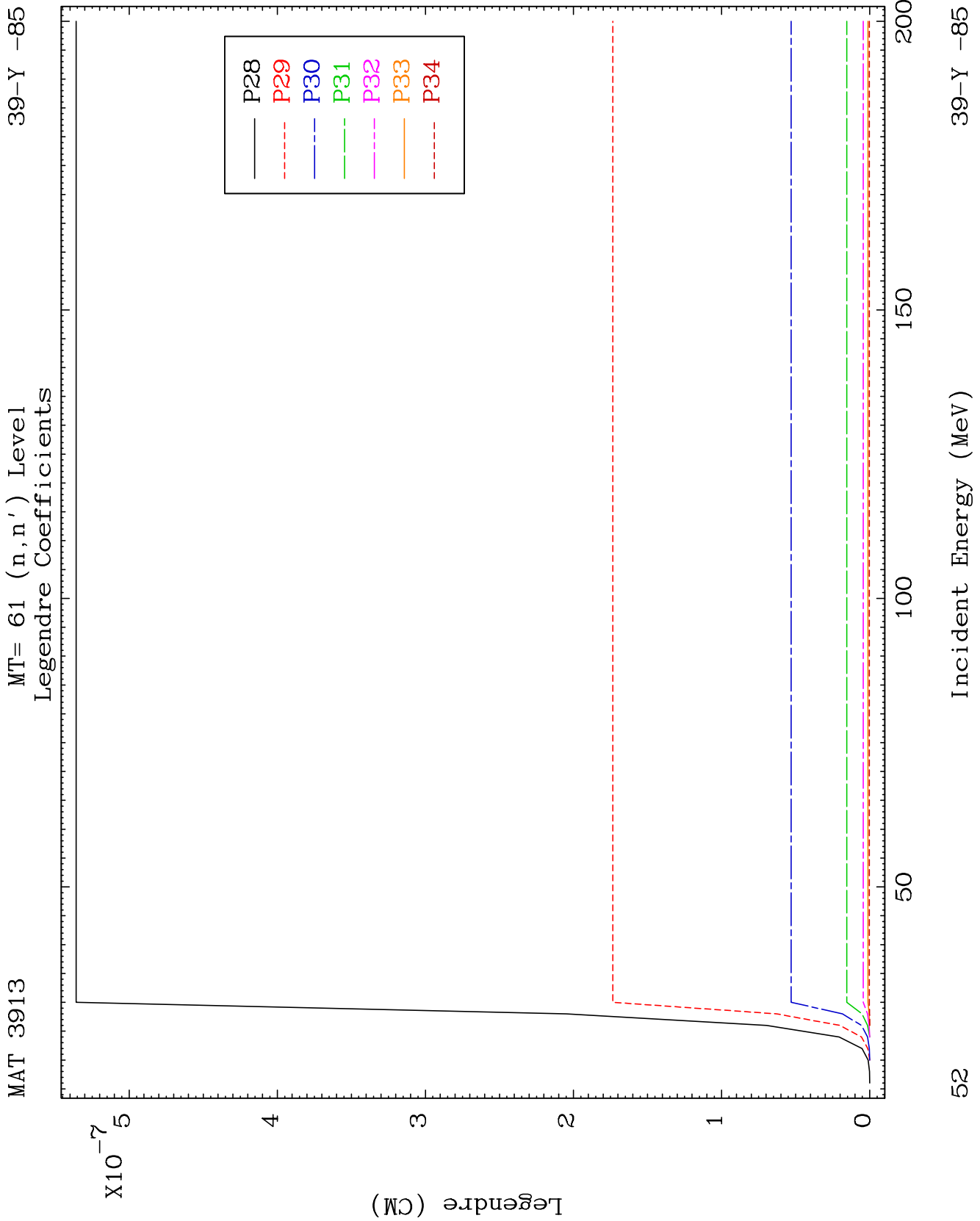
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Legendre Coefficients

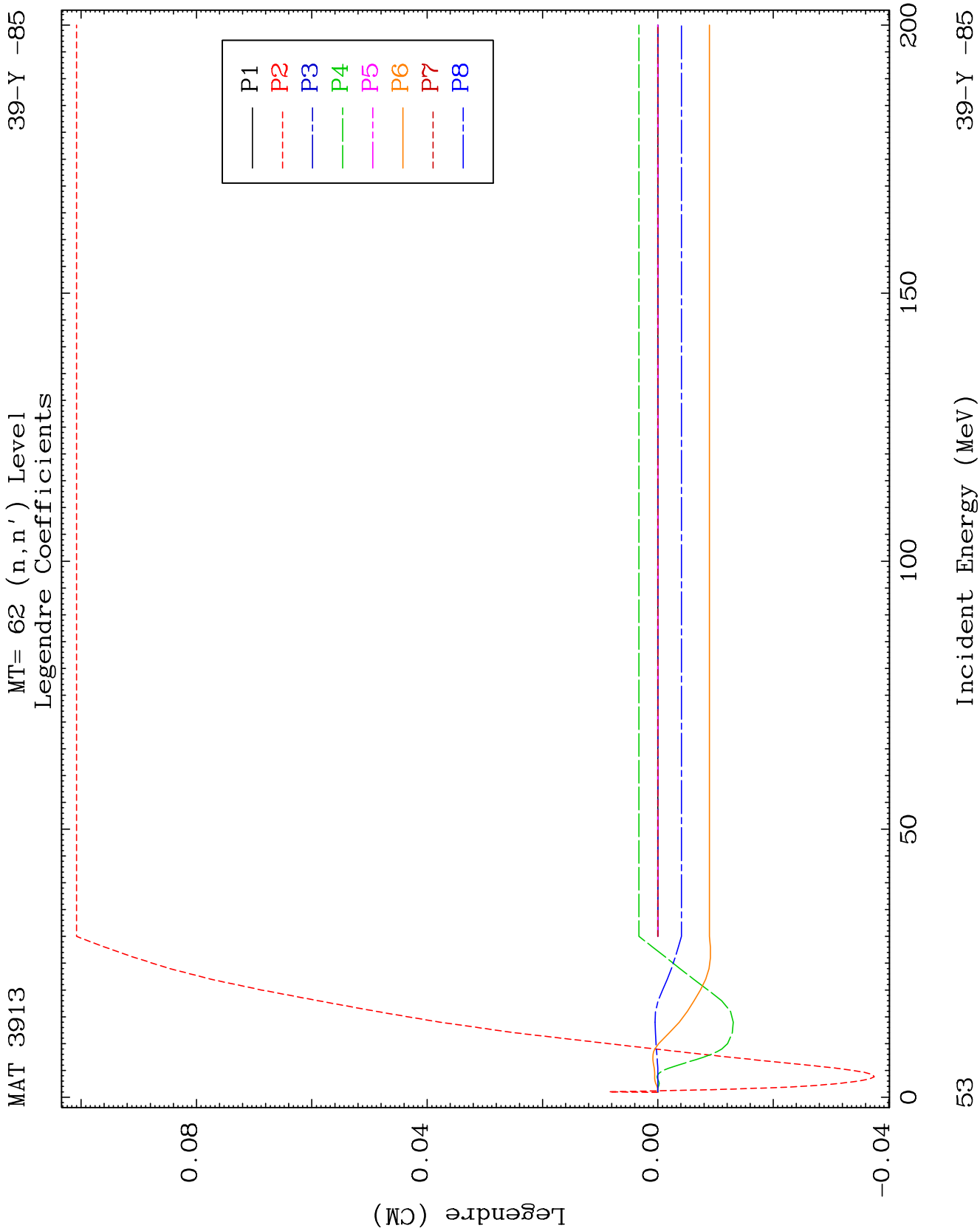
39-Y -85

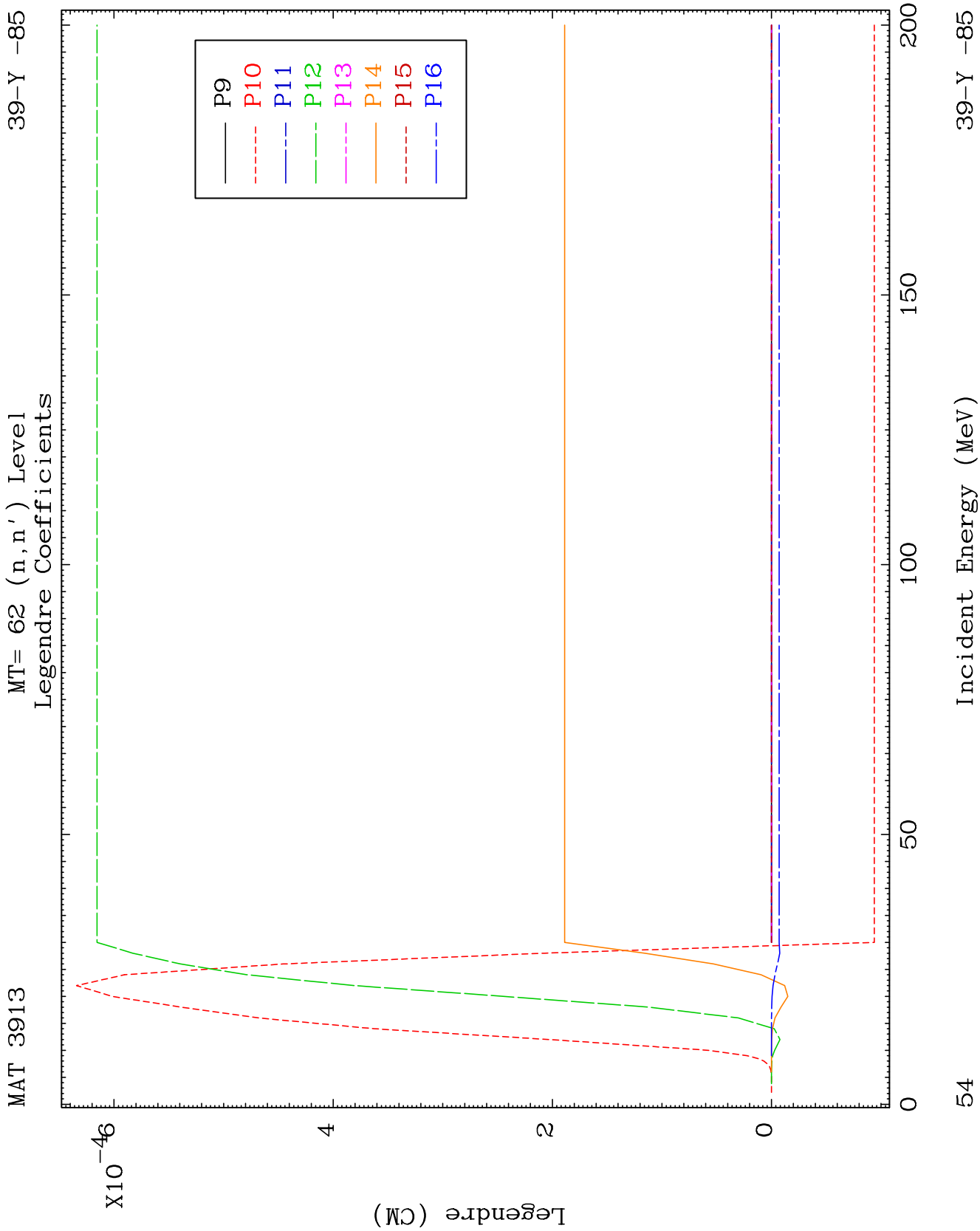


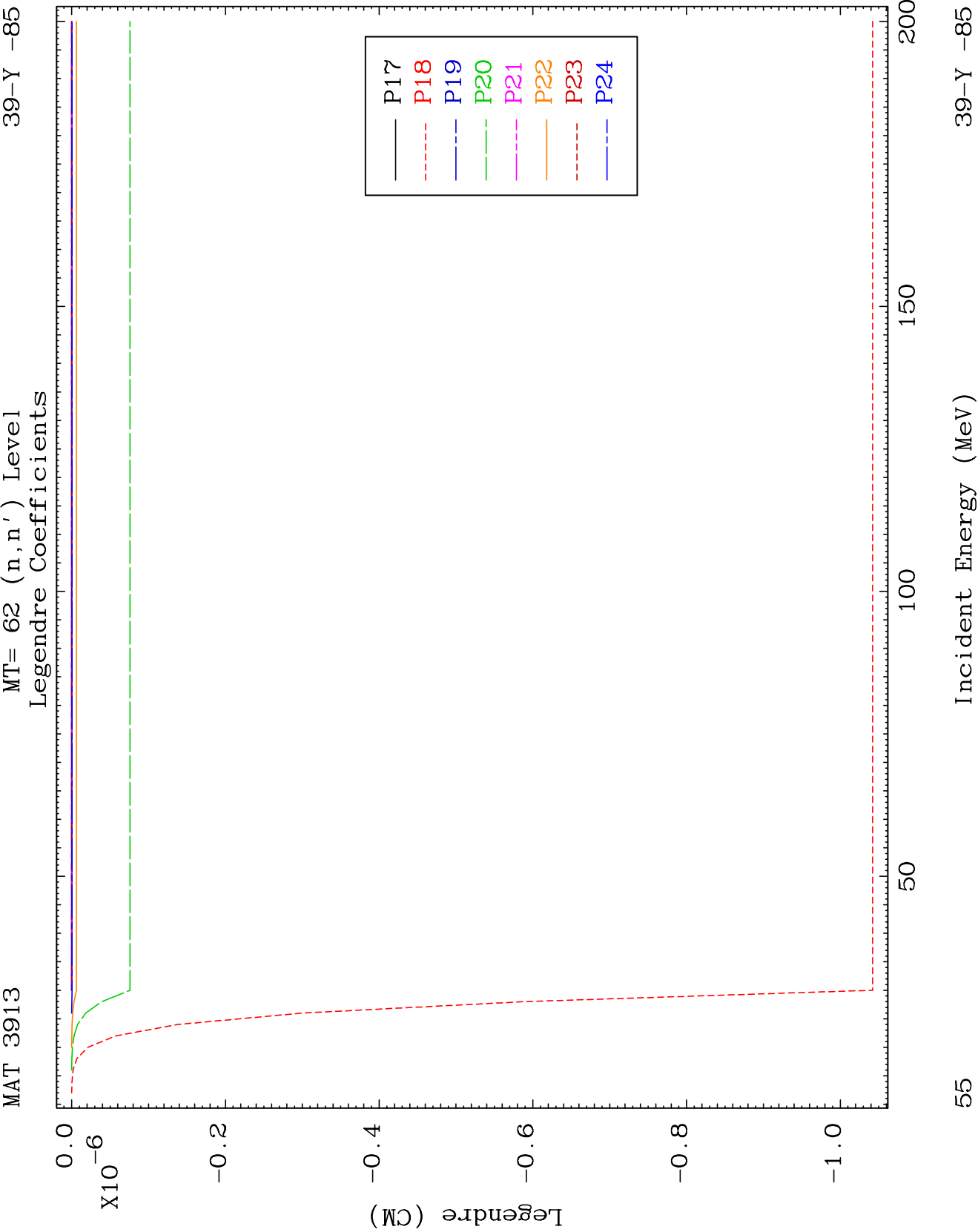








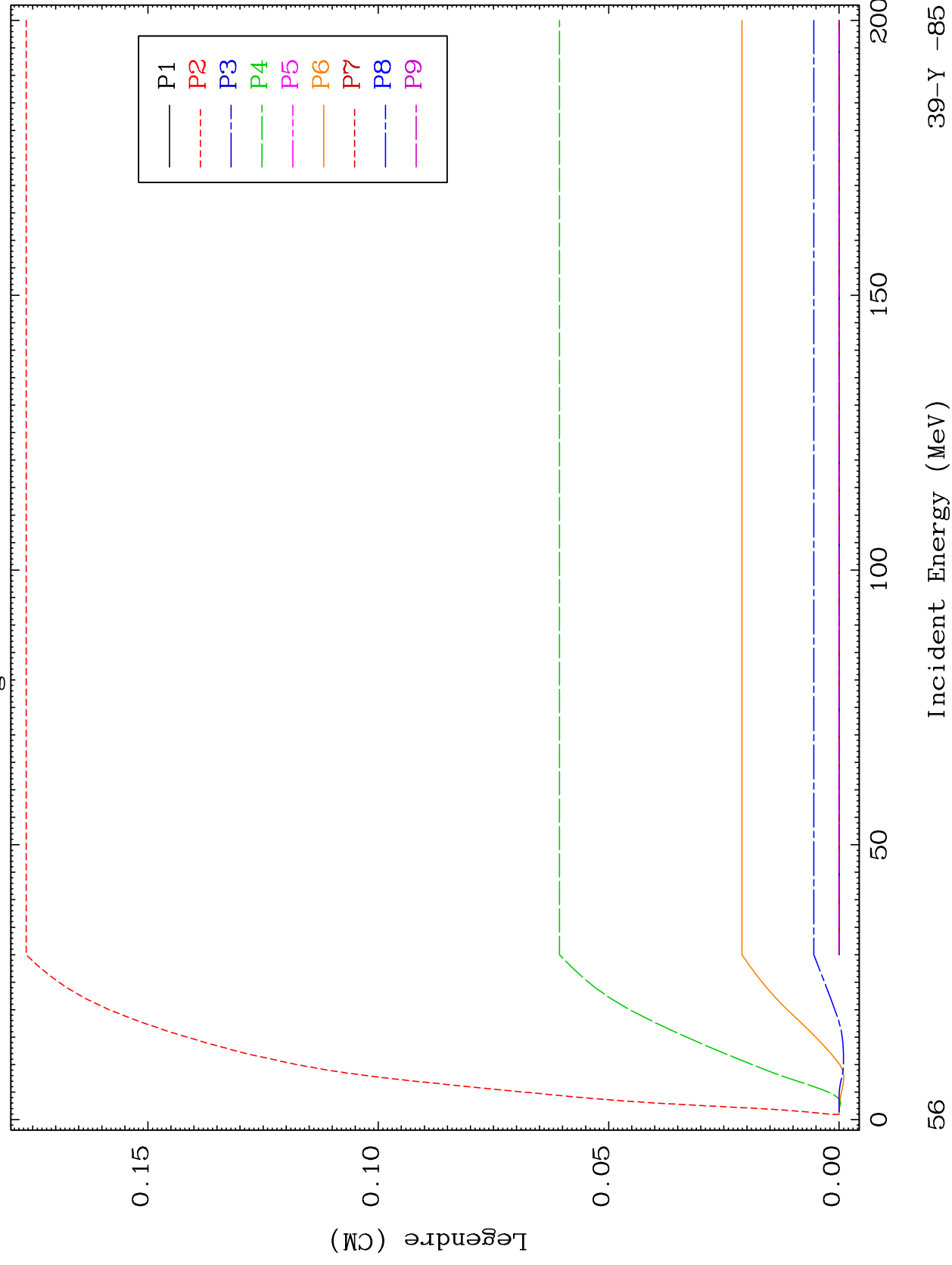




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MT= 63 (n, n') Level
Legendre Coefficients

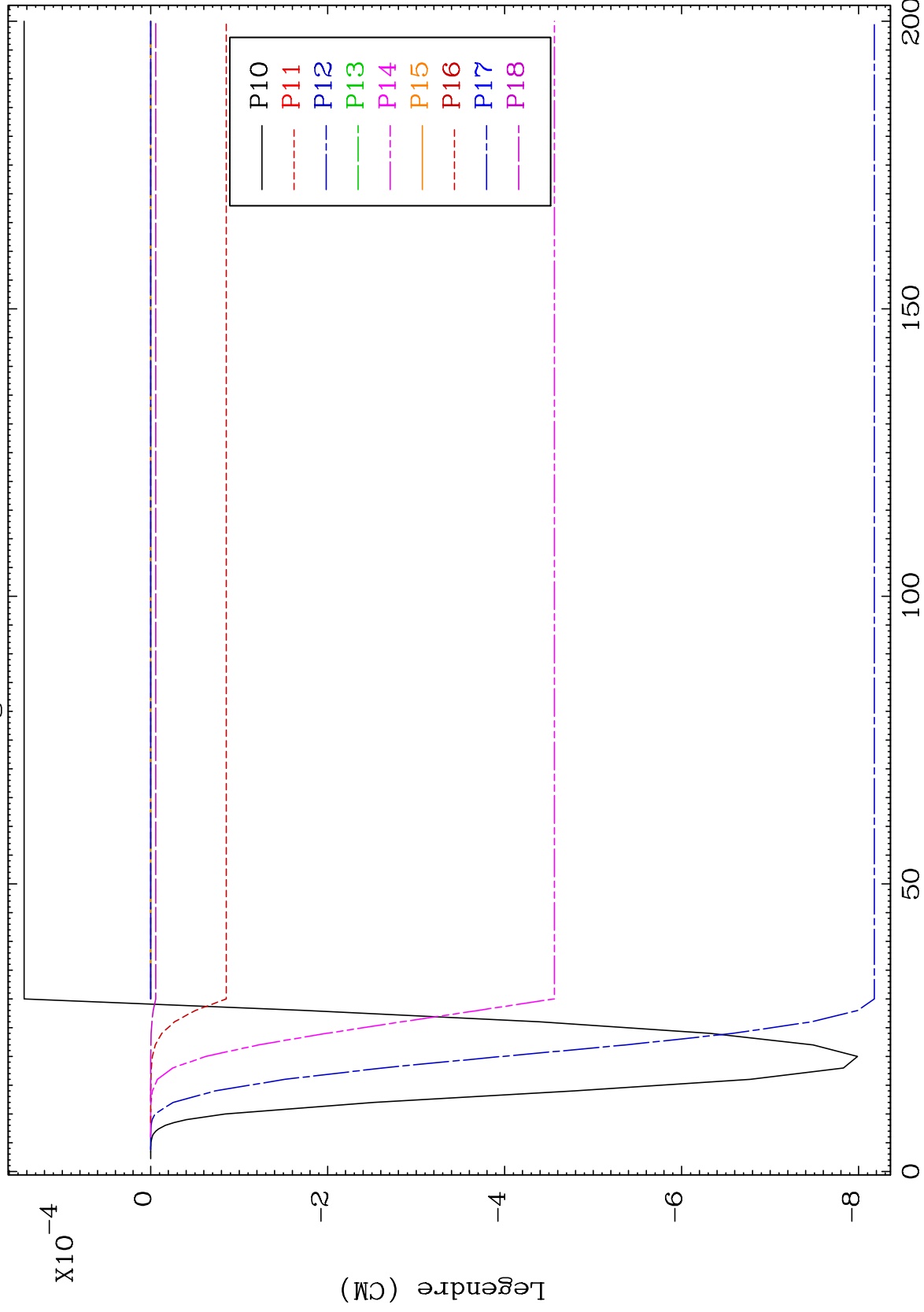
39-Y -85



MAT 3913

MT= 63 (n,n') Level
Legendre Coefficients

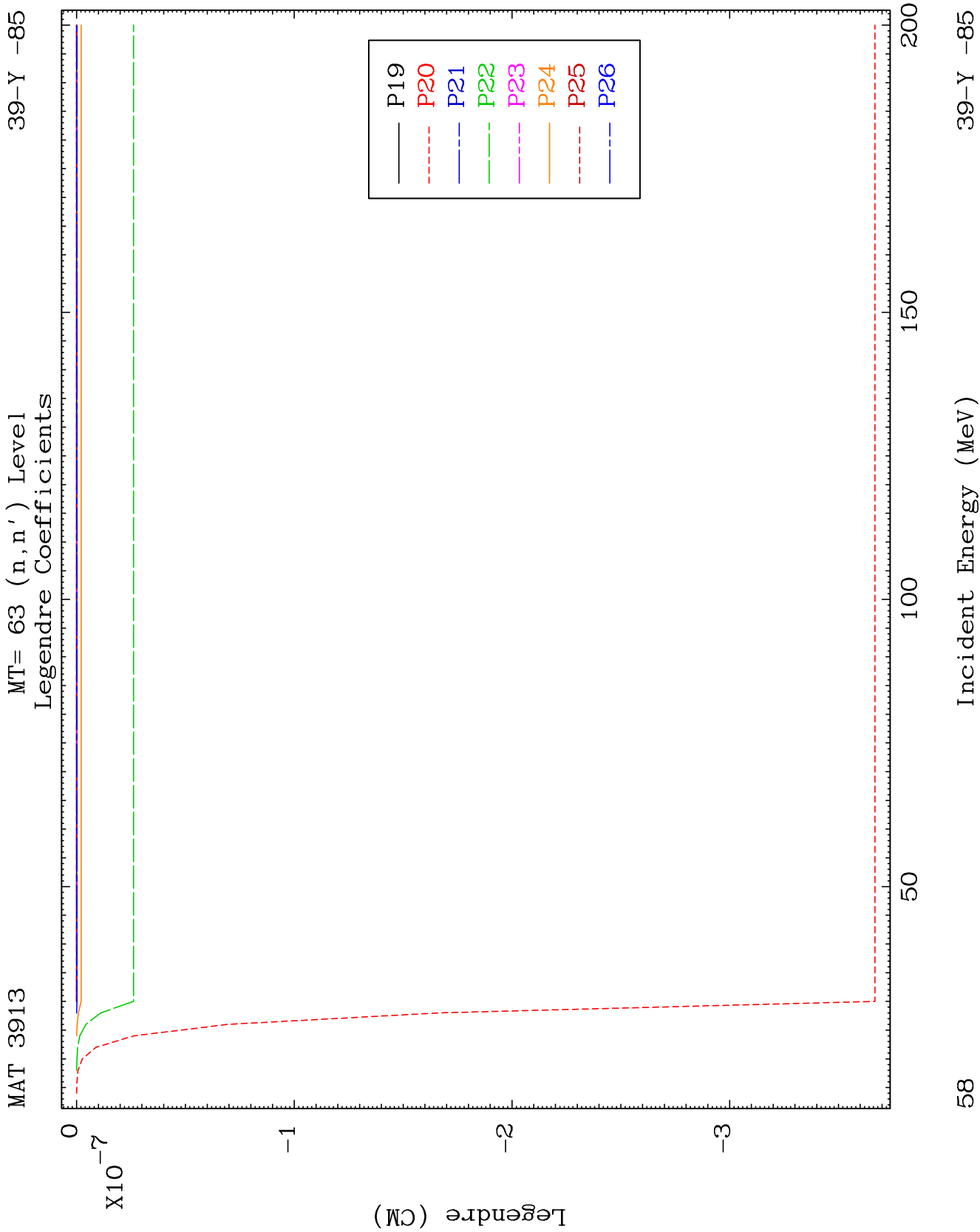
39-Y -85

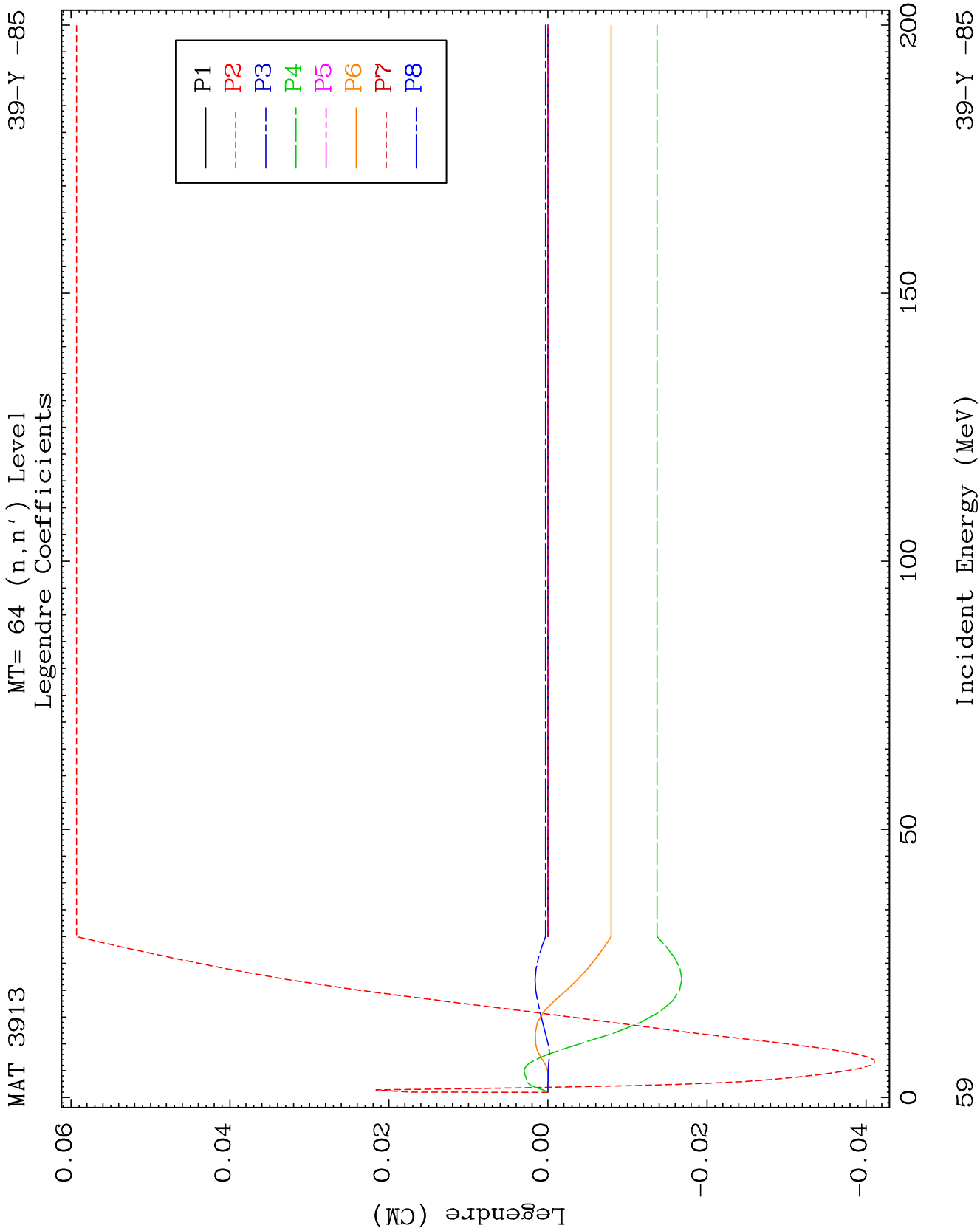


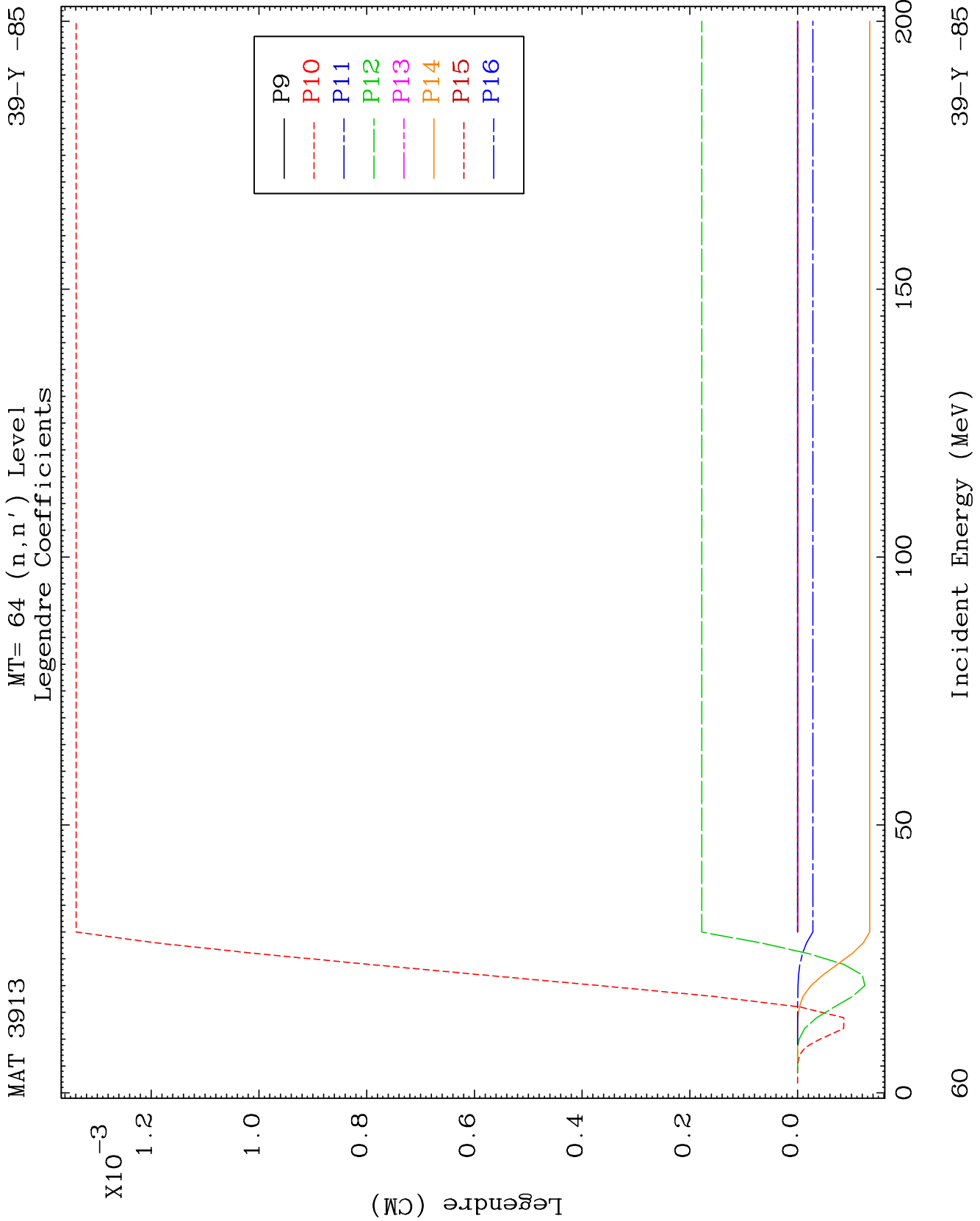
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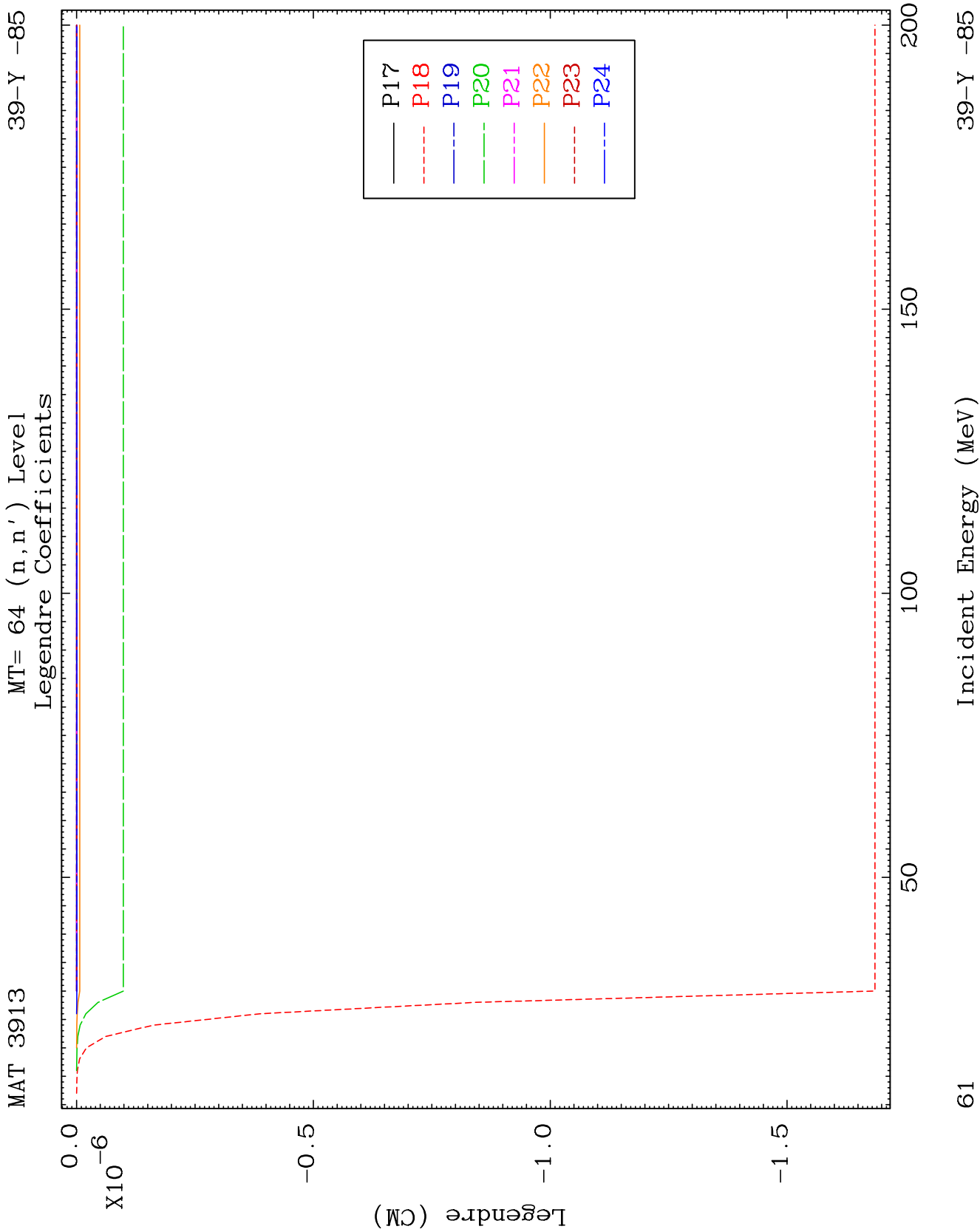
Incident Energy (MeV)

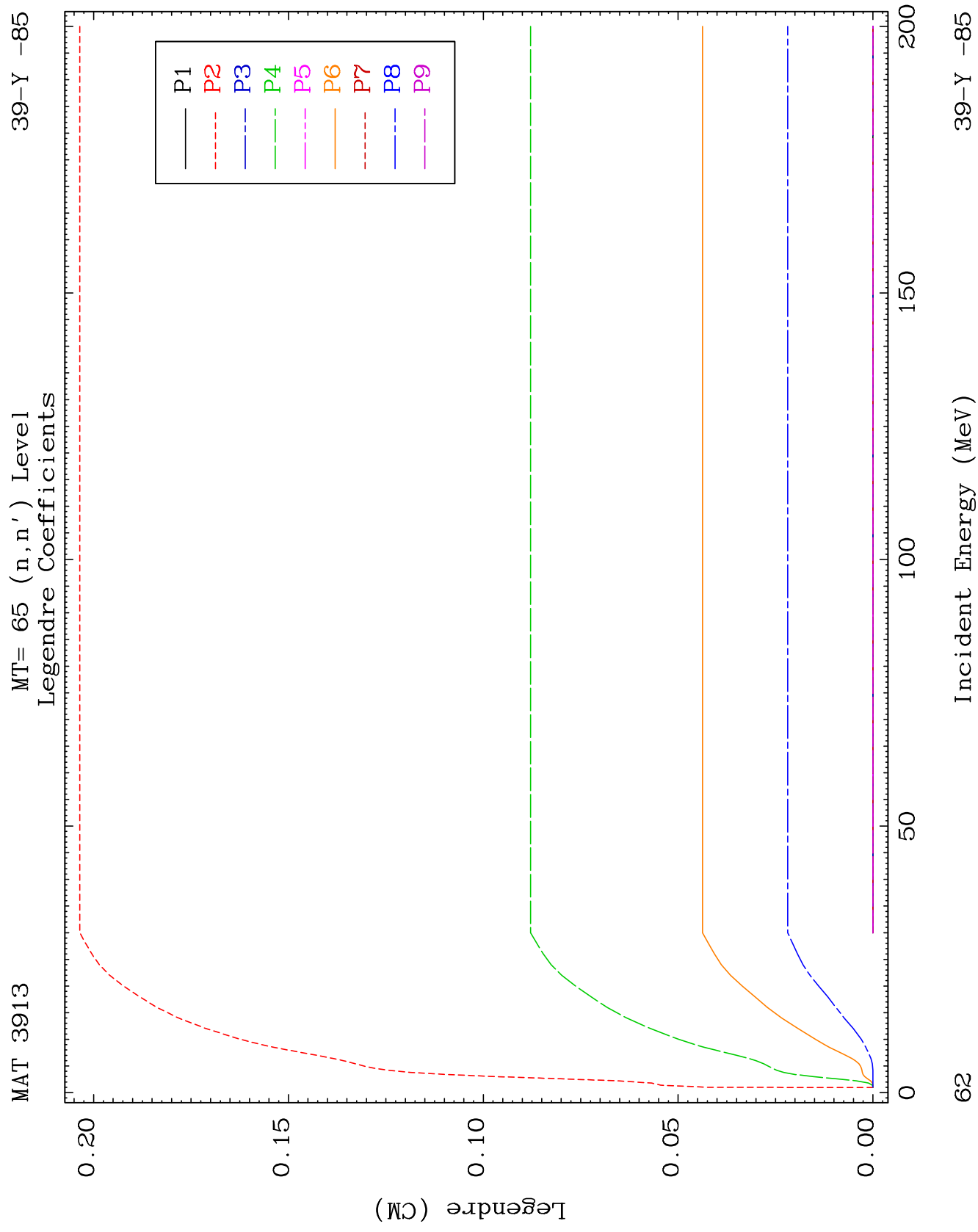
39-Y -85

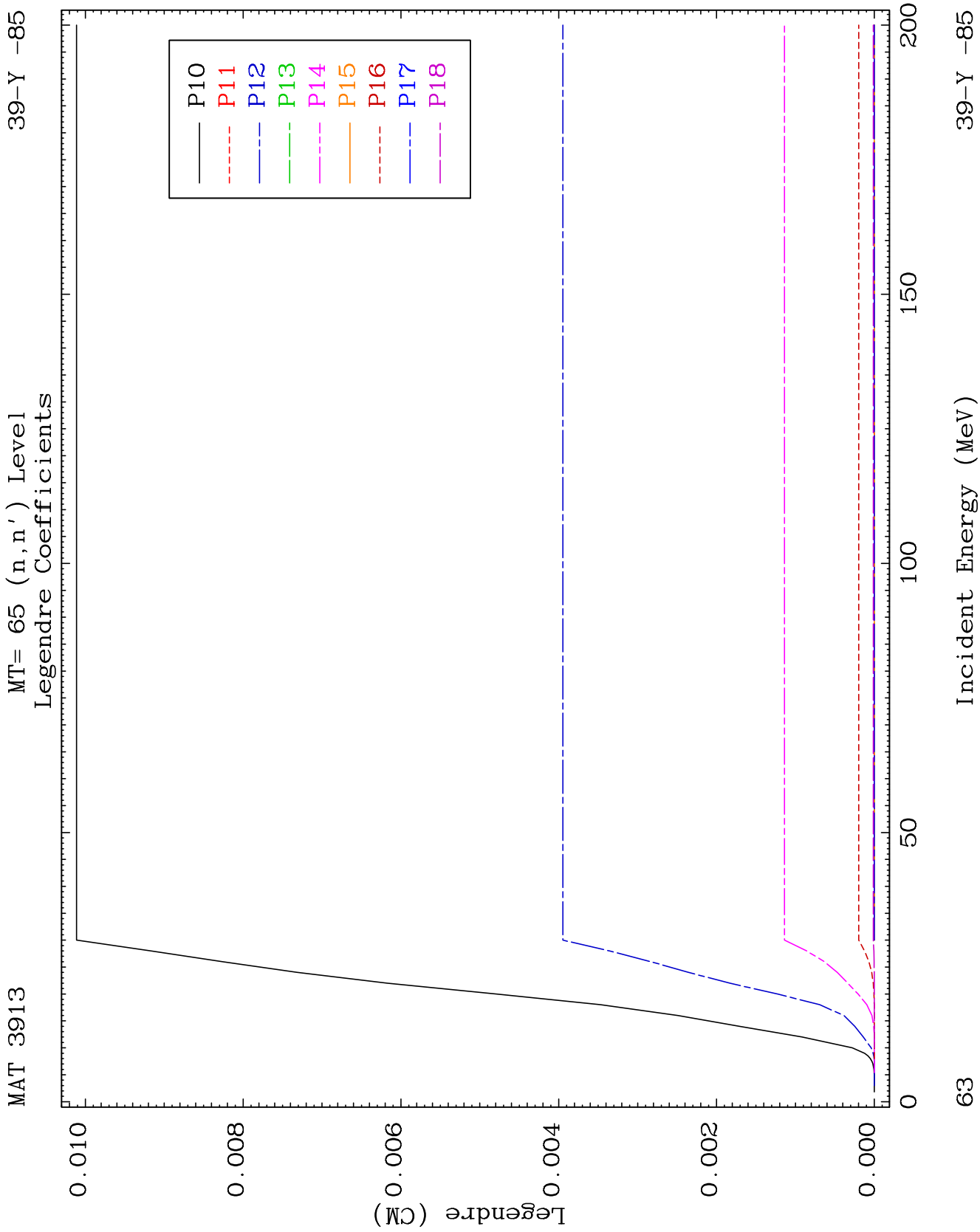


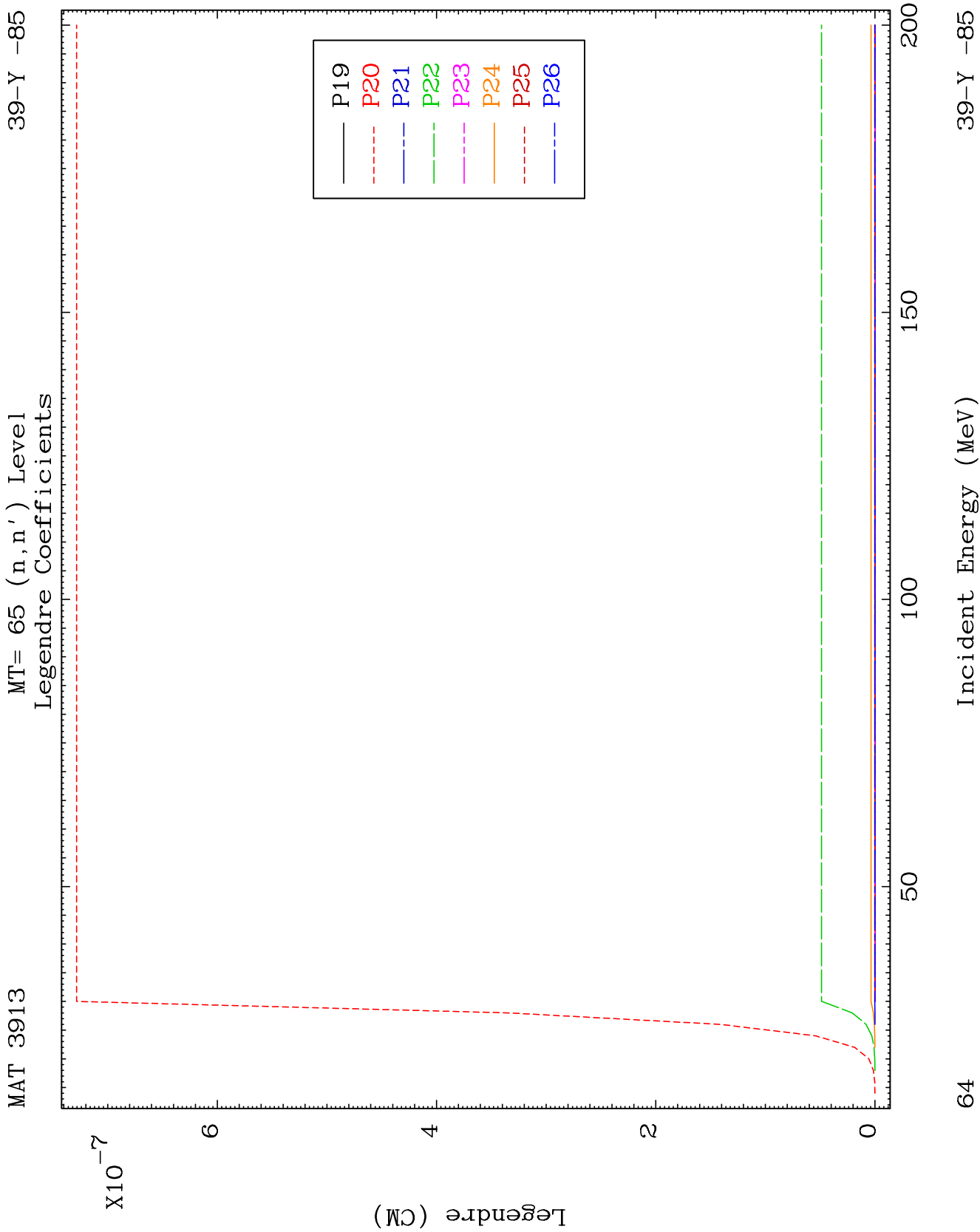


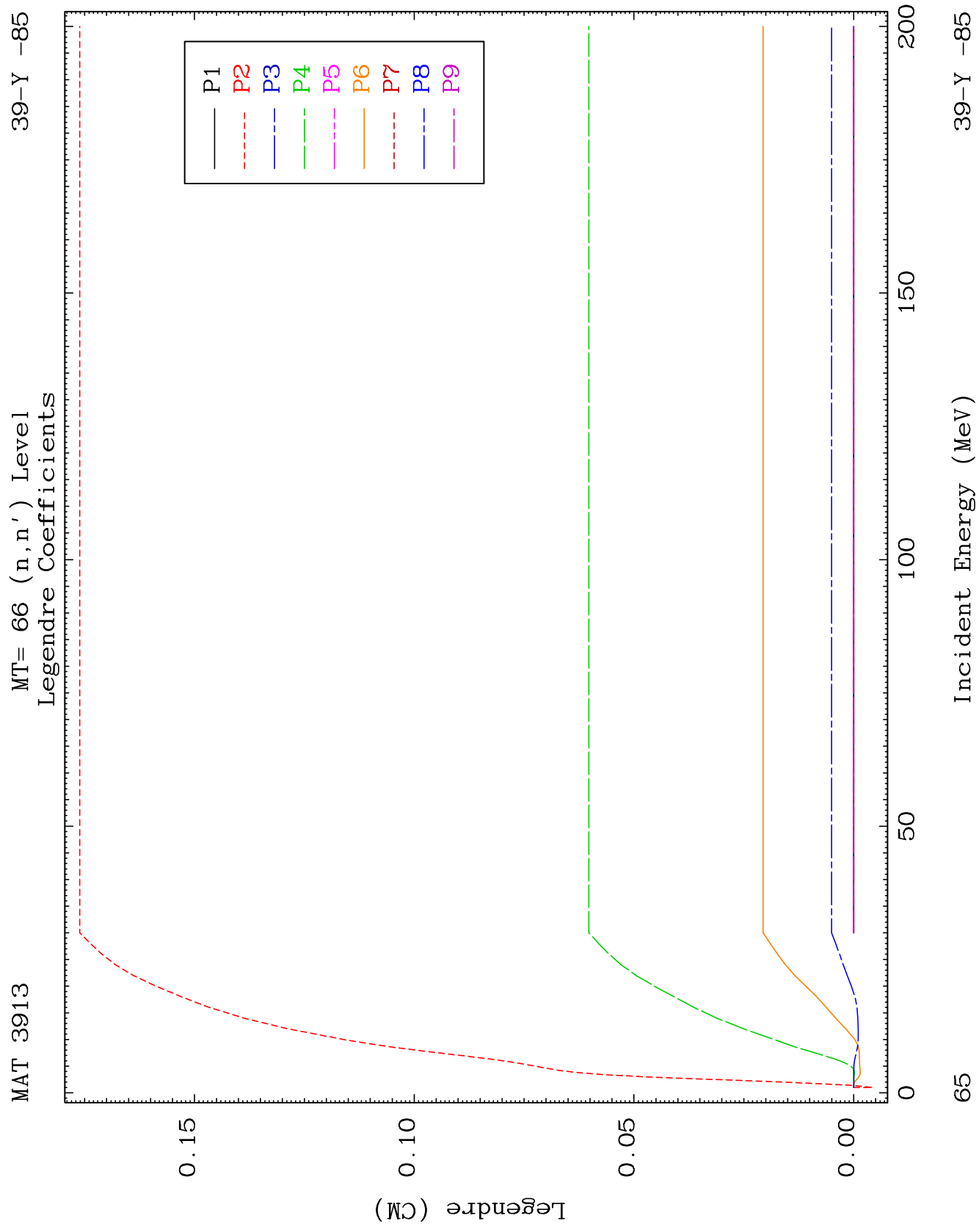


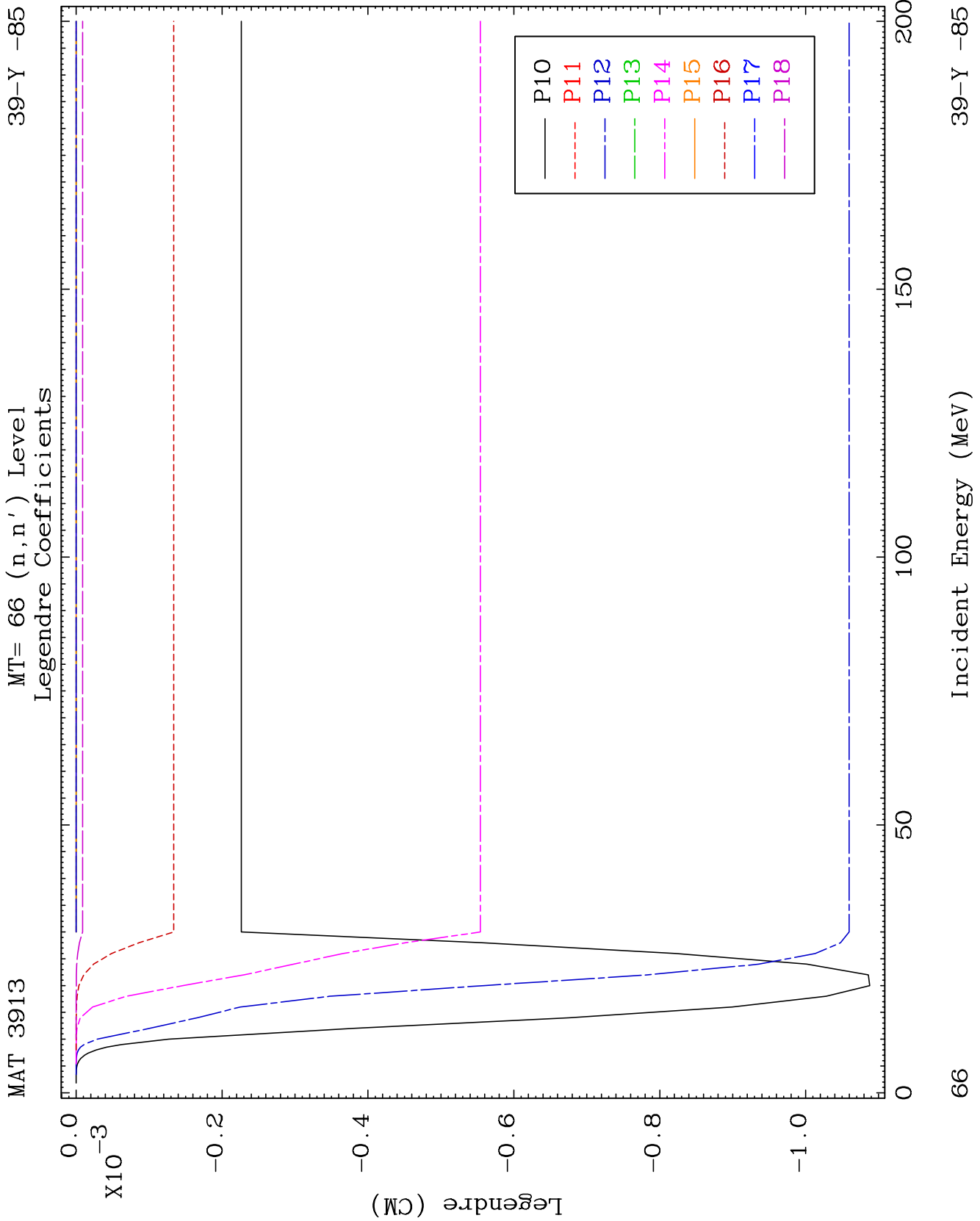


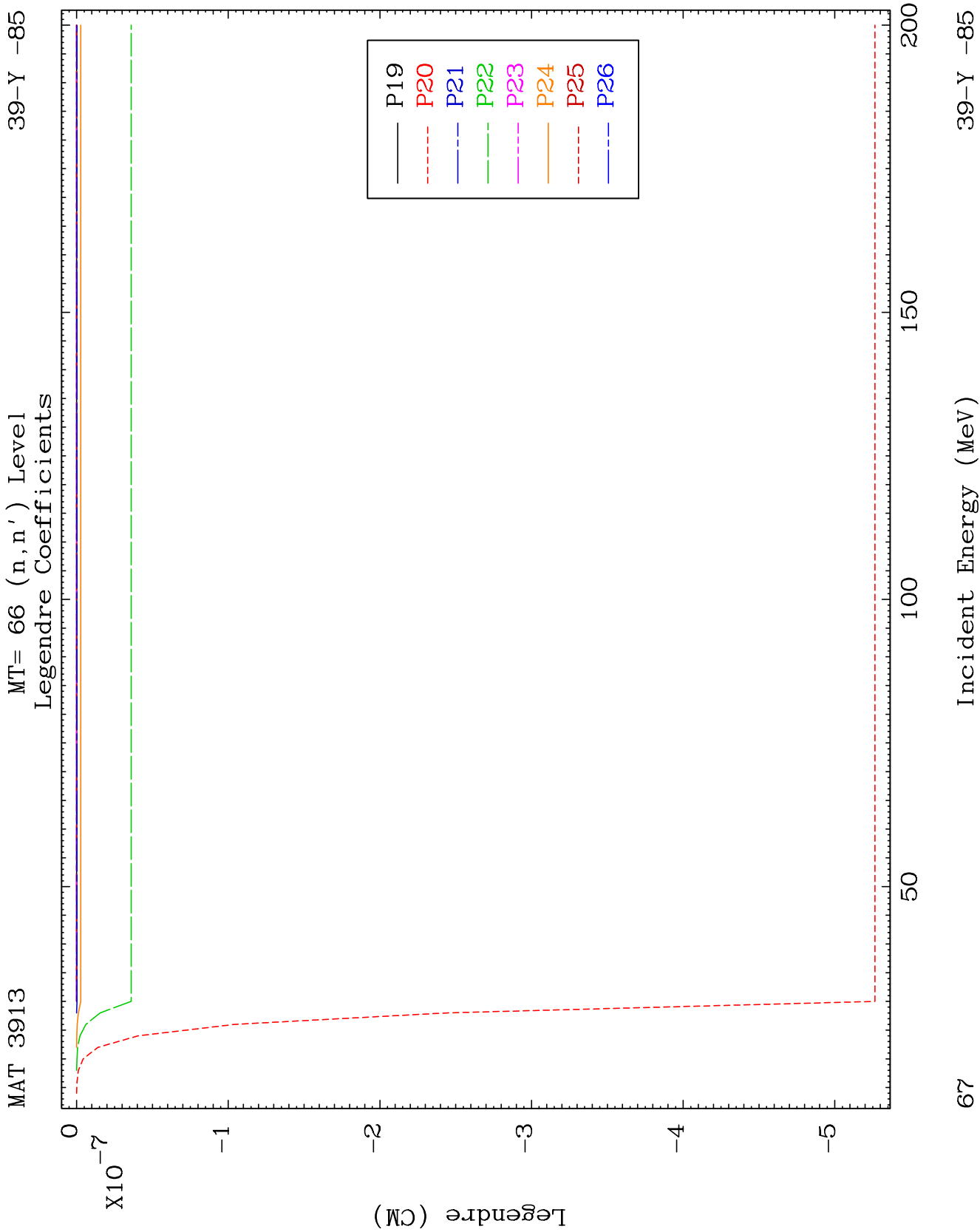


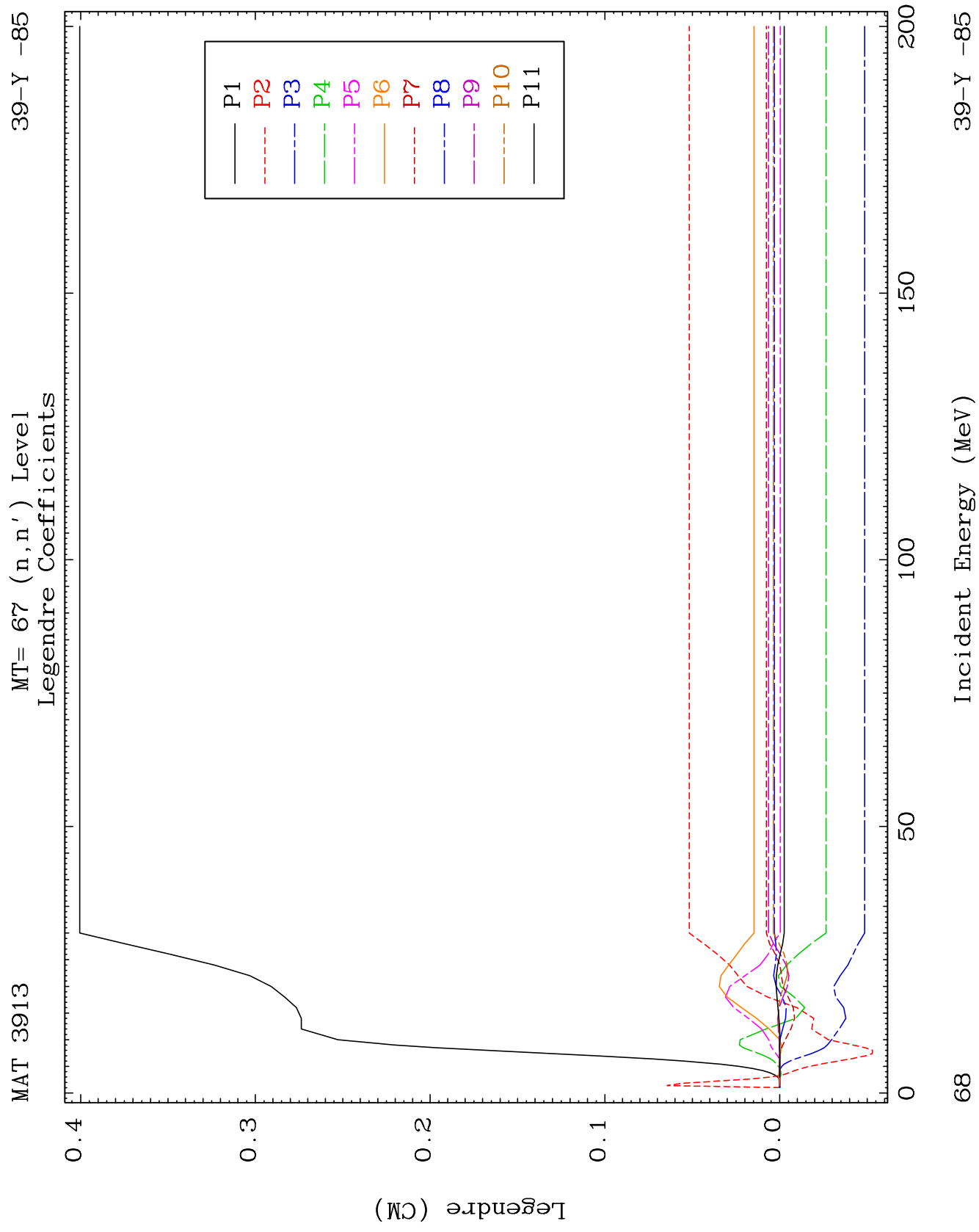


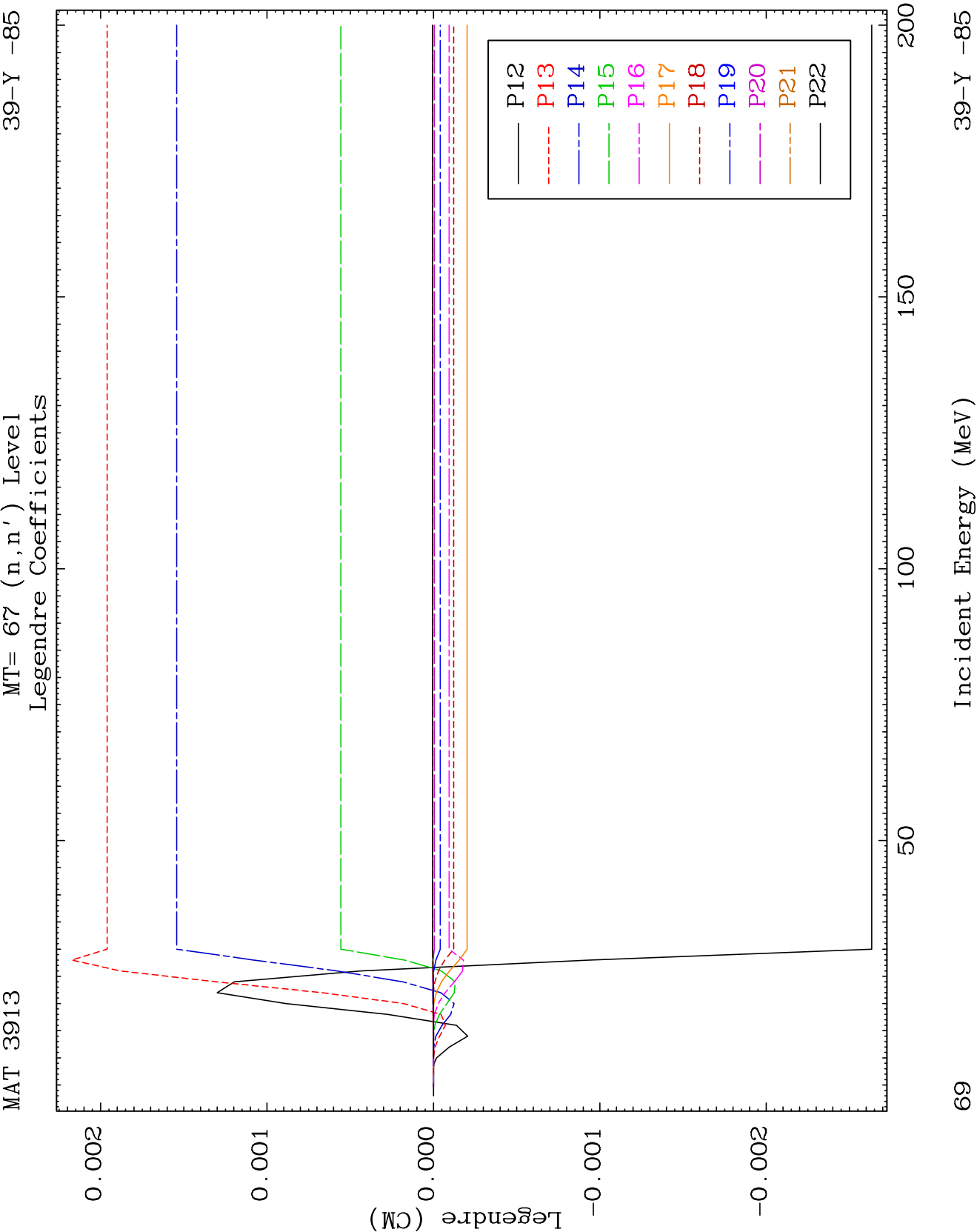


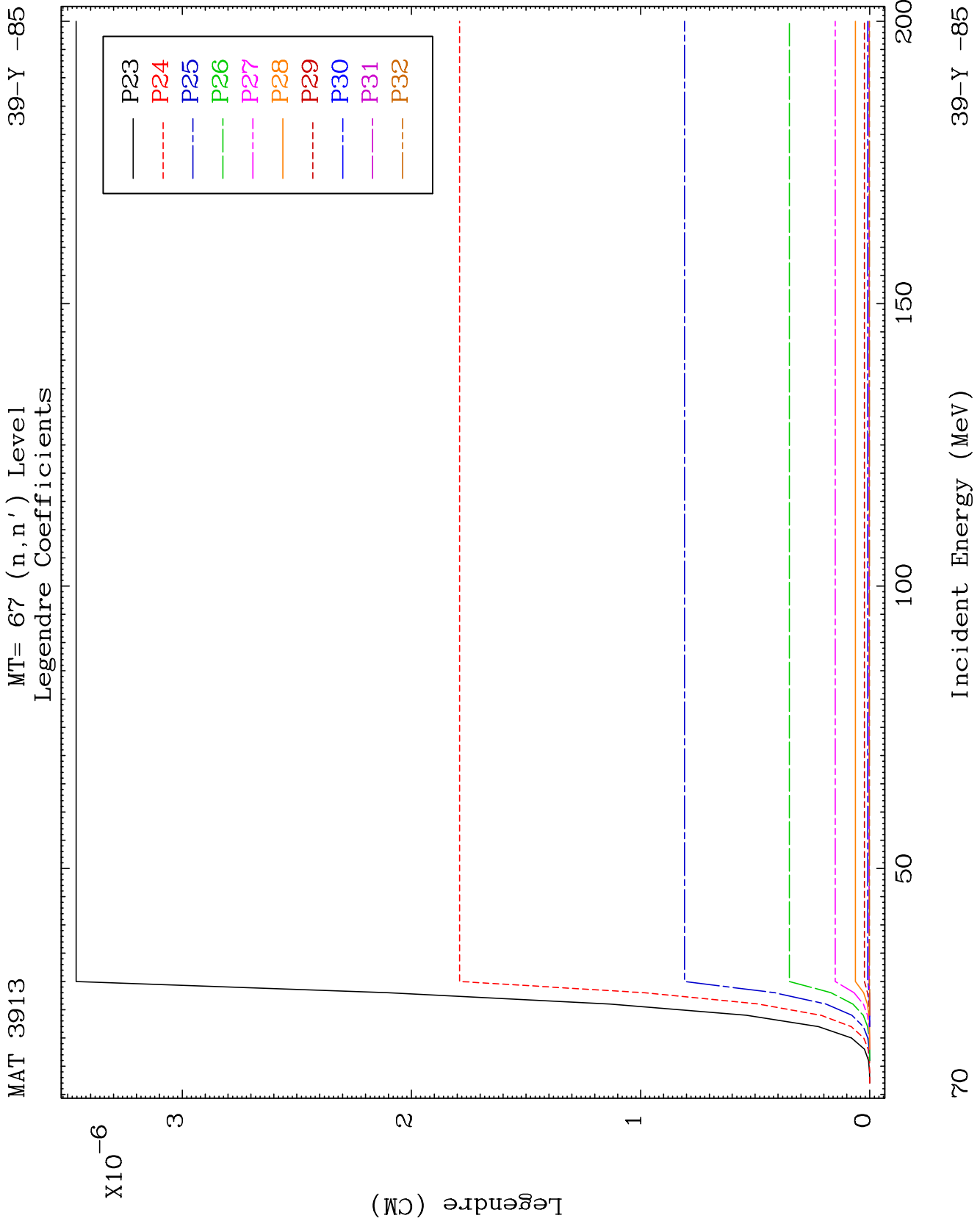








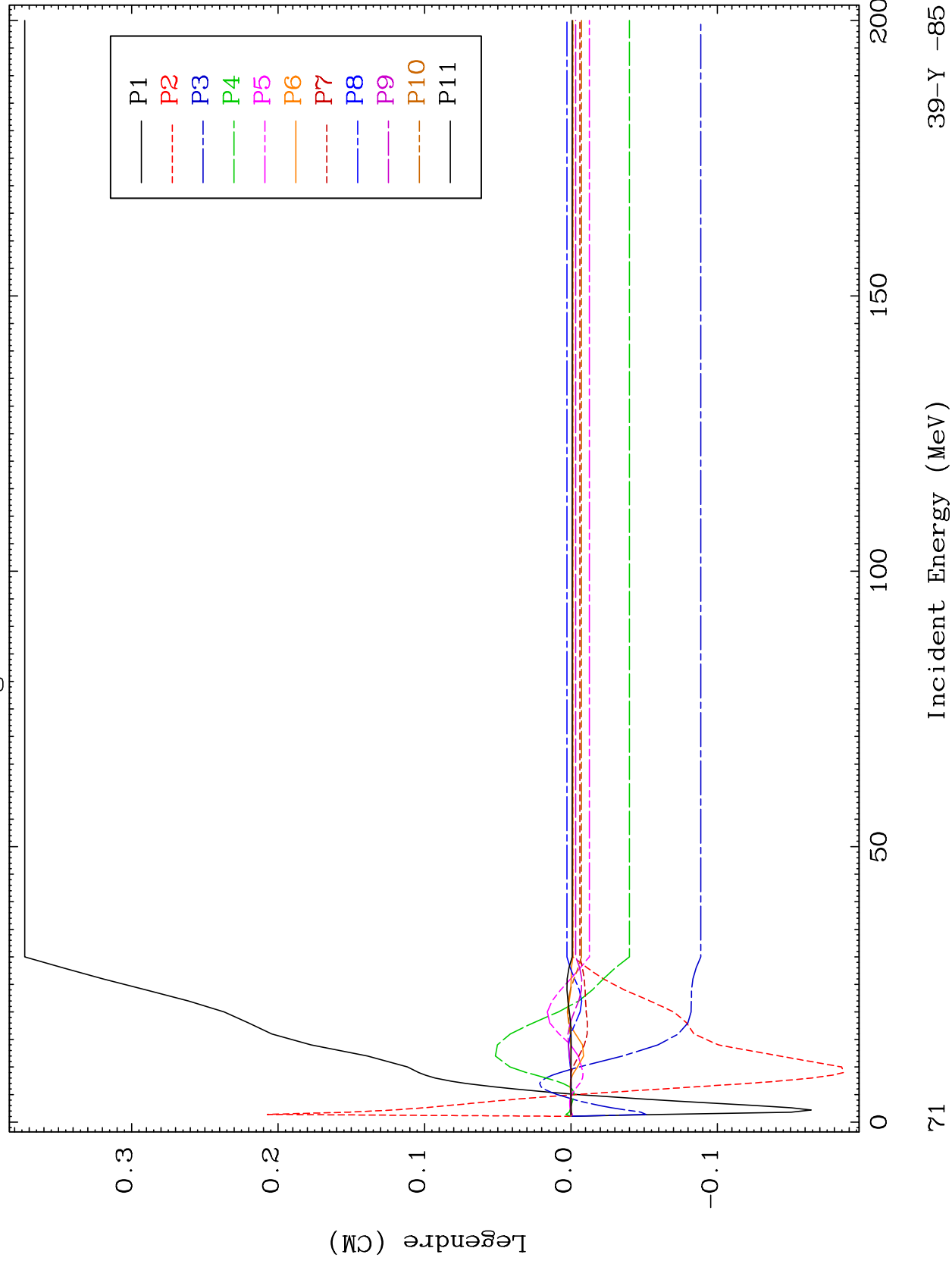


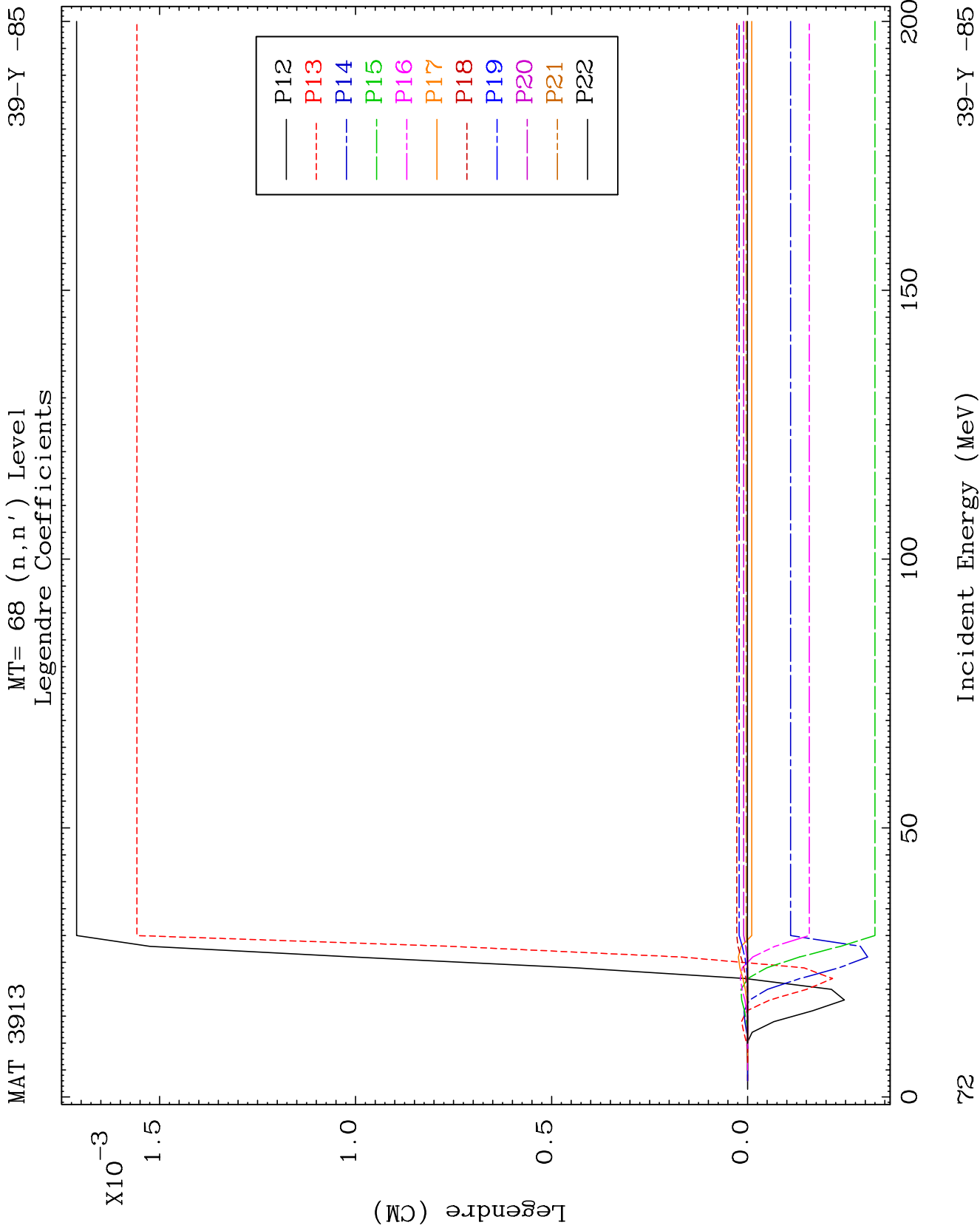


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MT= 68 (n, n') Level
Legendre Coefficients

39-Y -85

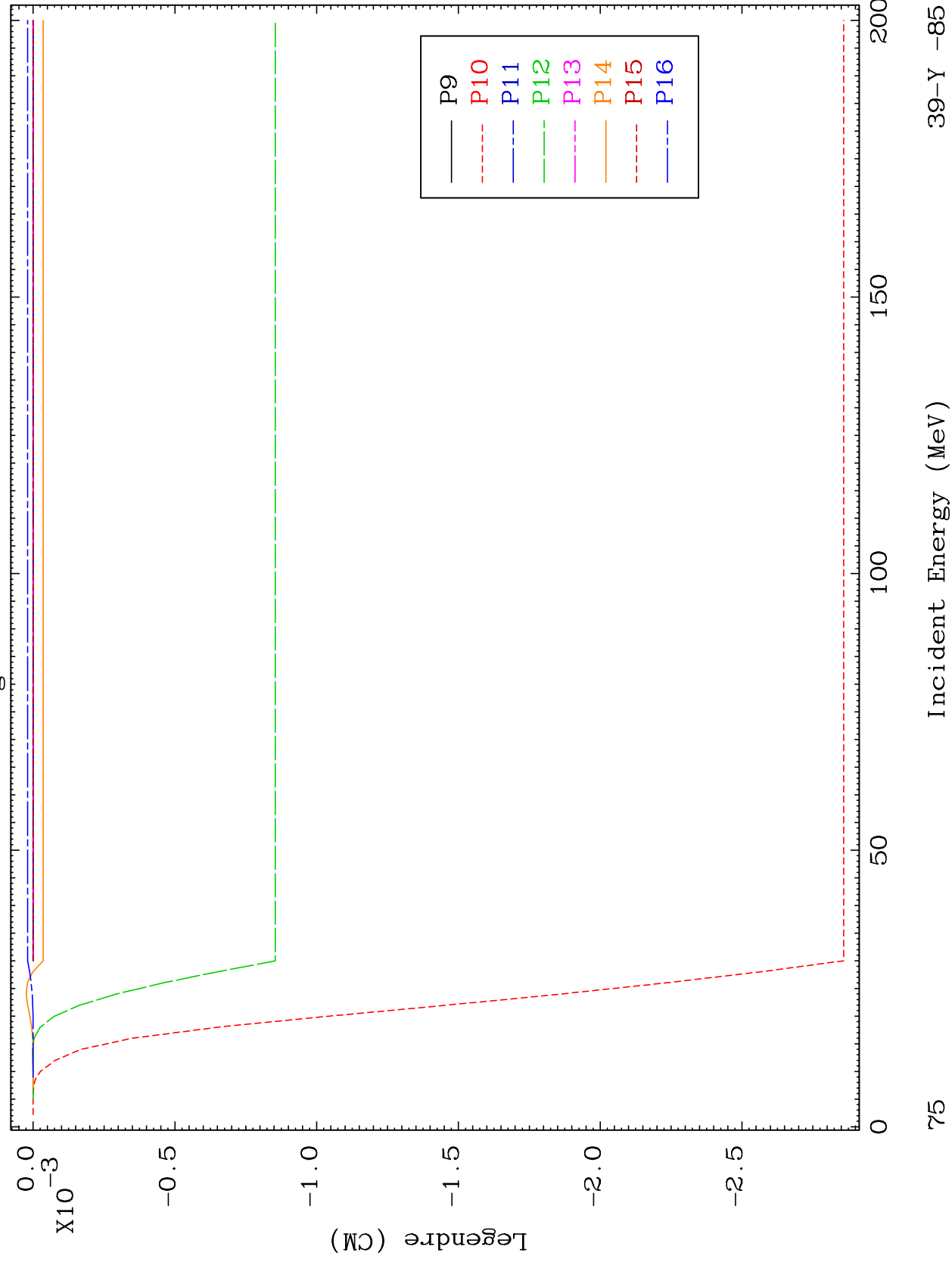


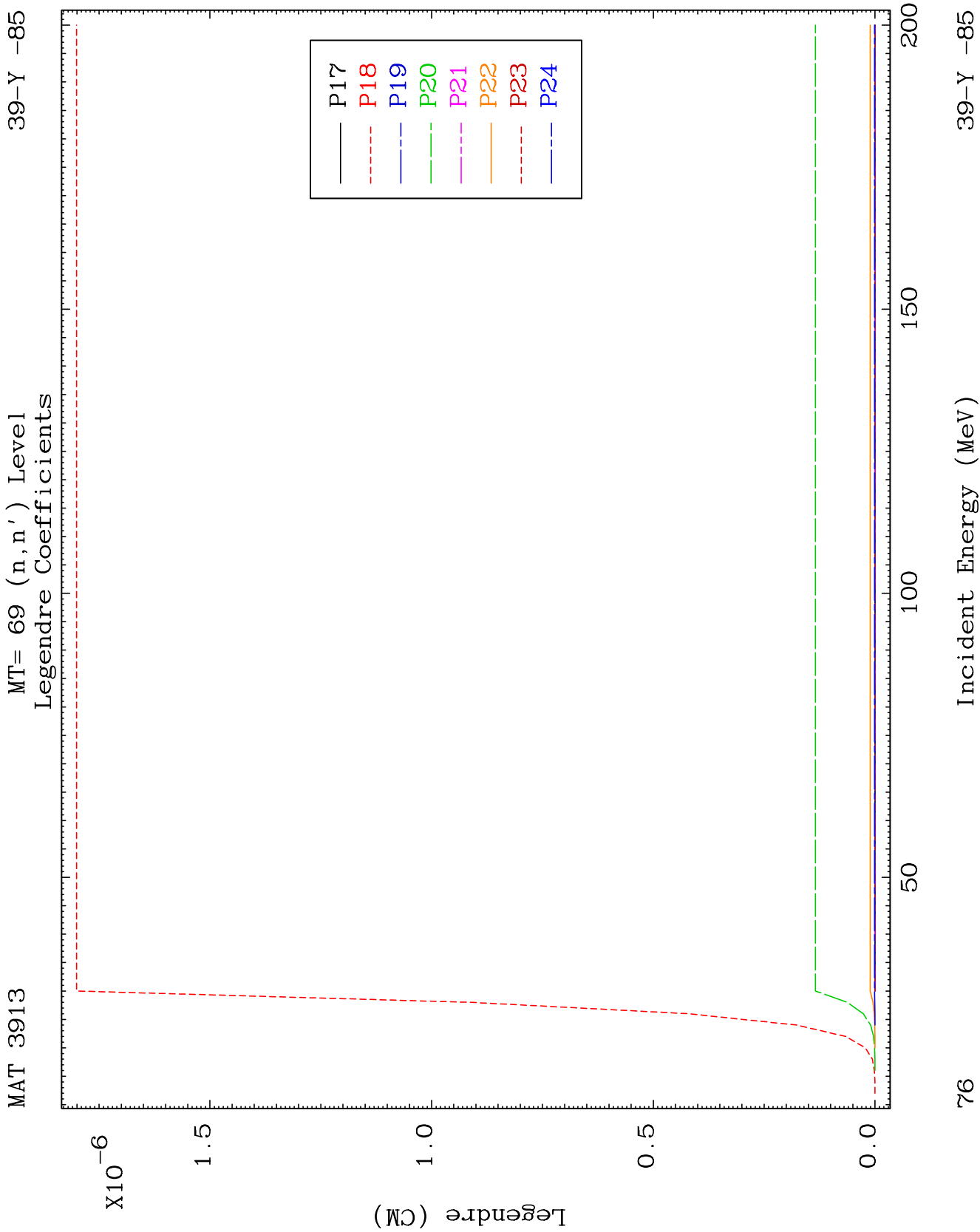


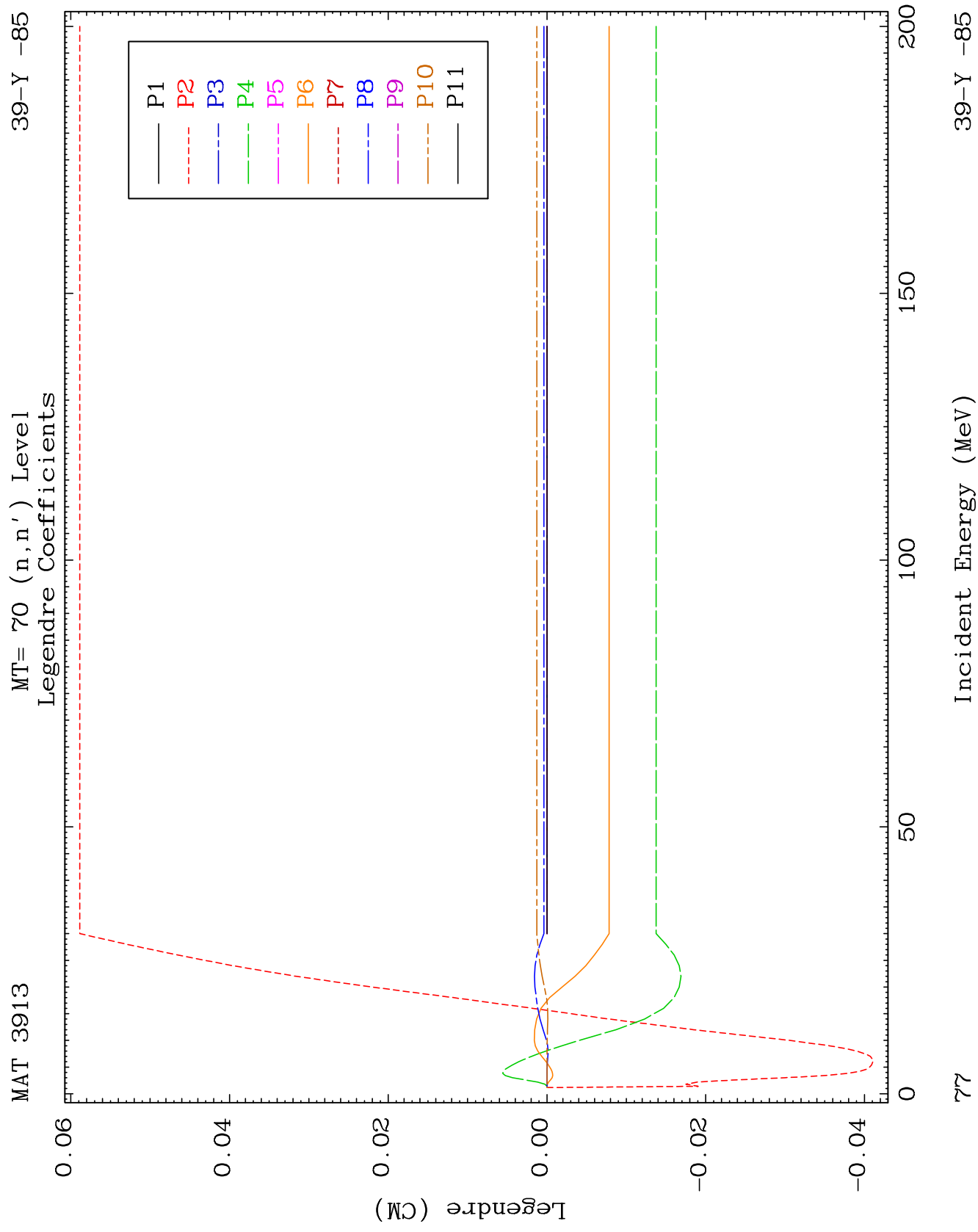
MAT 3913

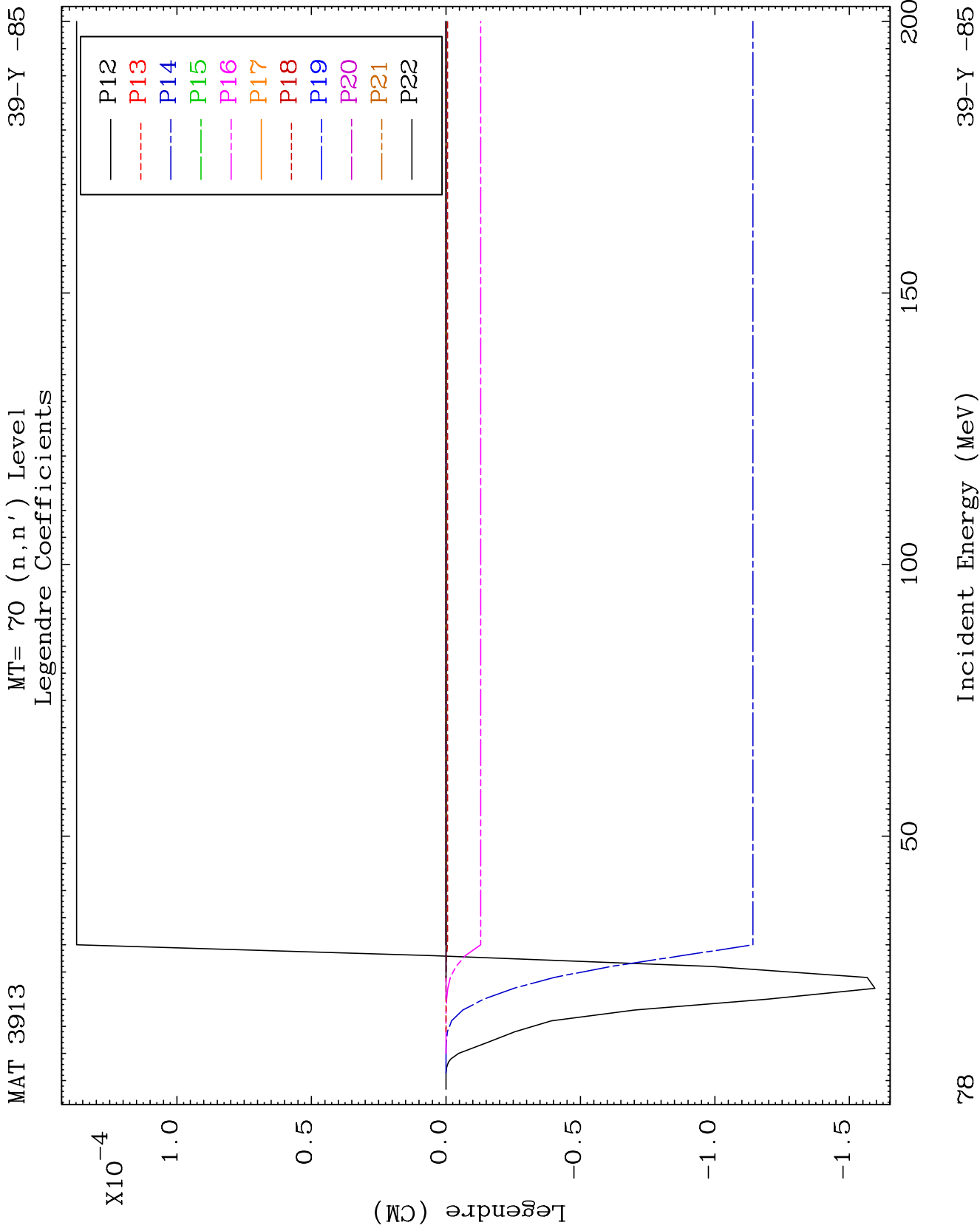
MT= 69 (n, n') Level
Legendre Coefficients

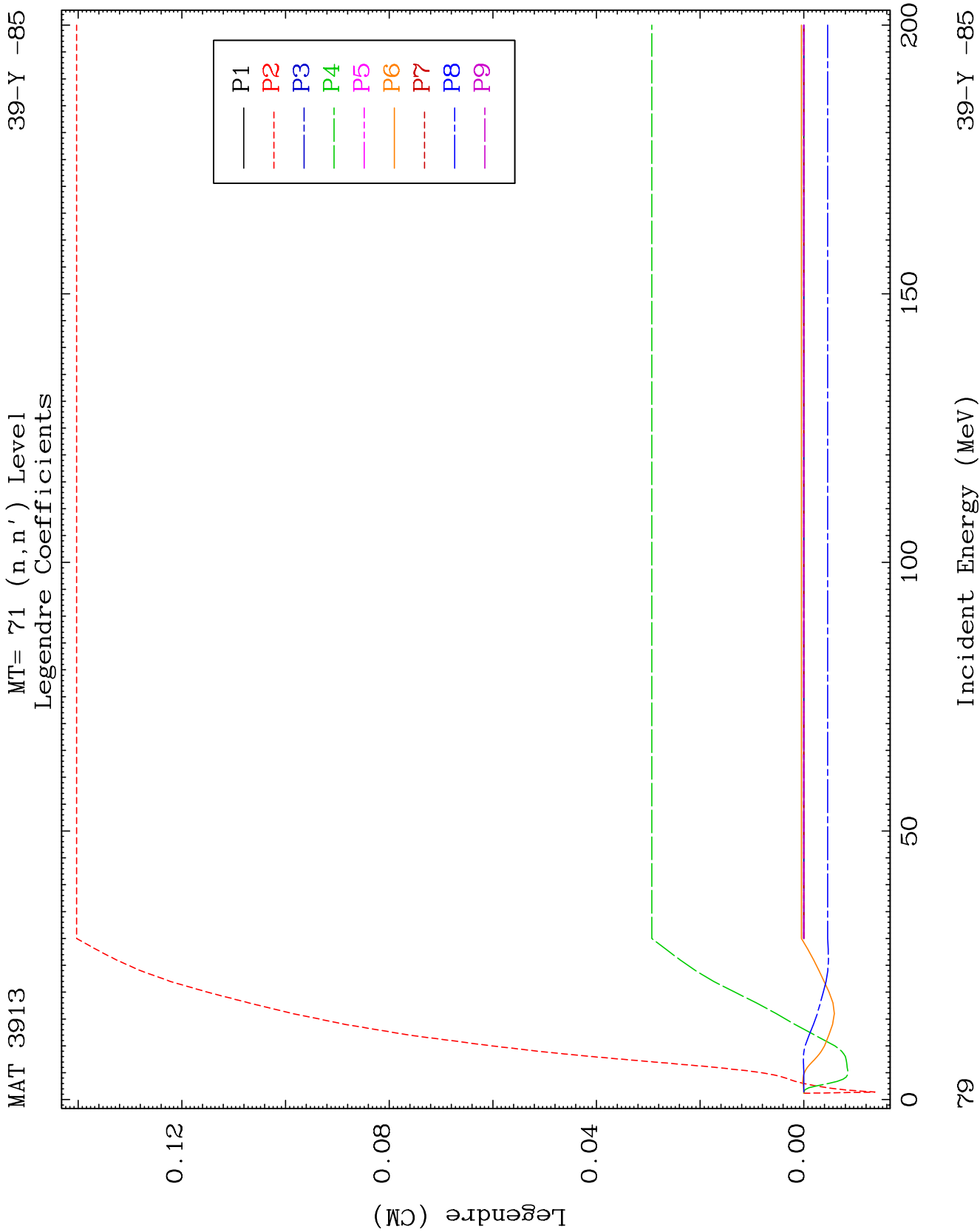
39-Y -85

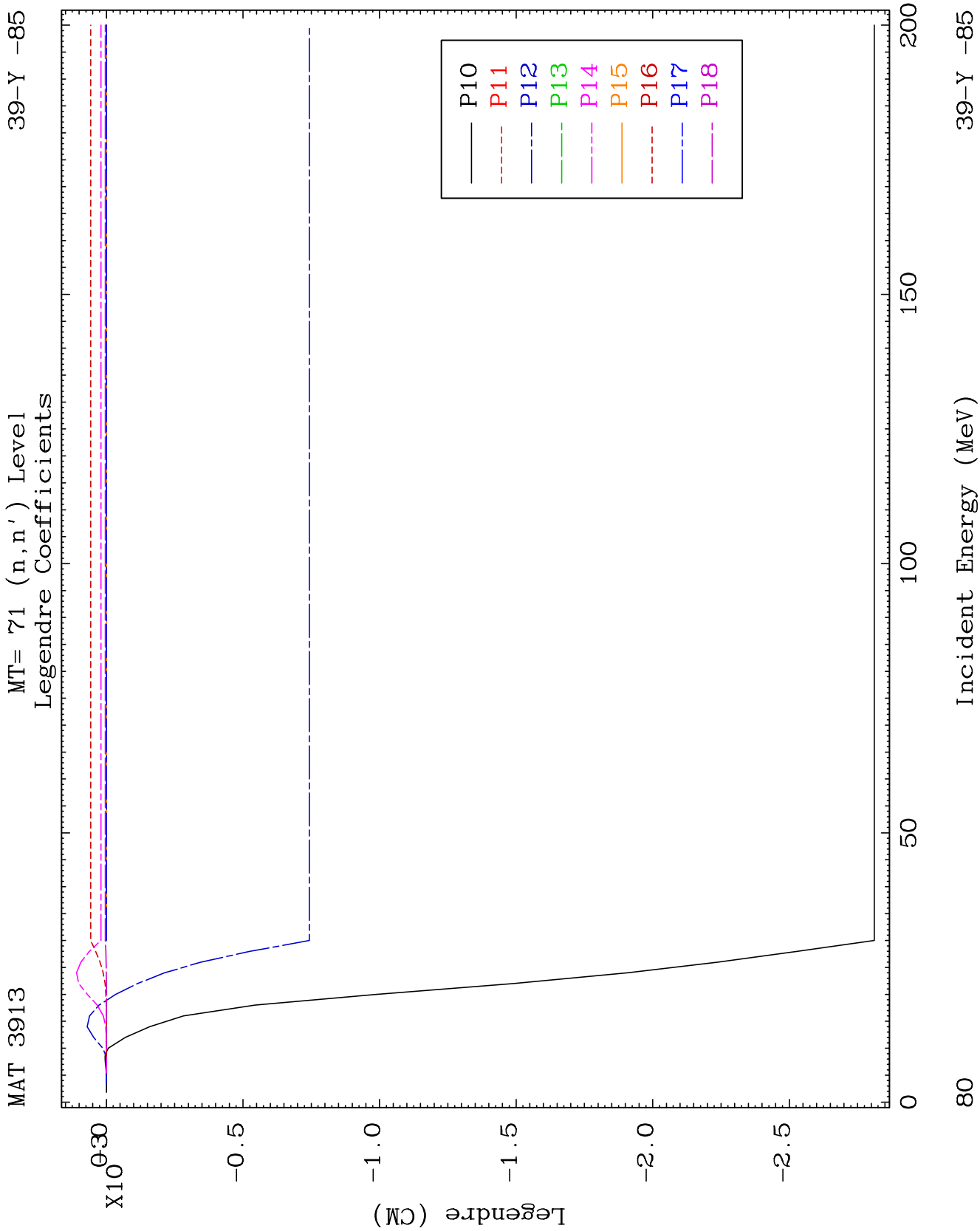


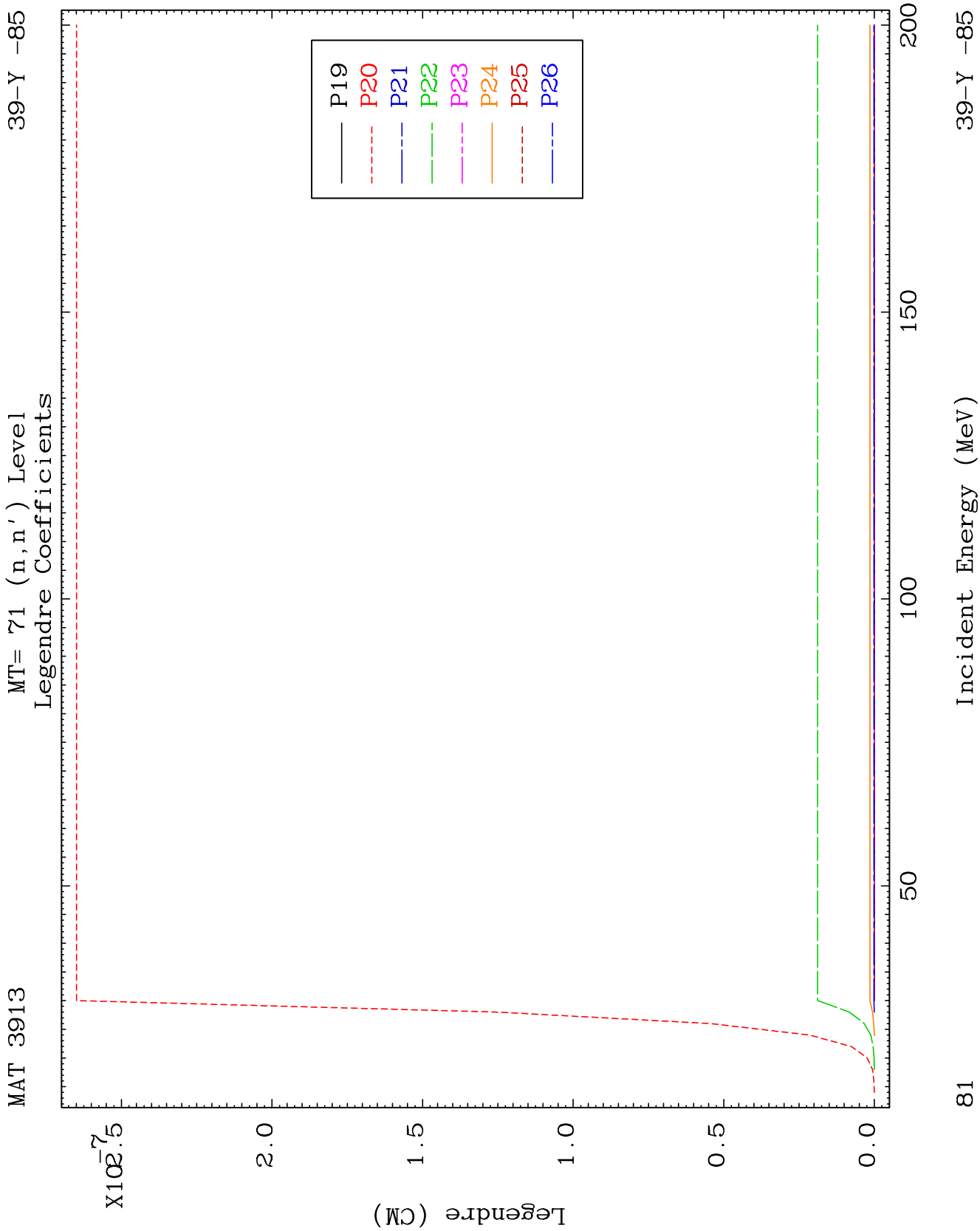








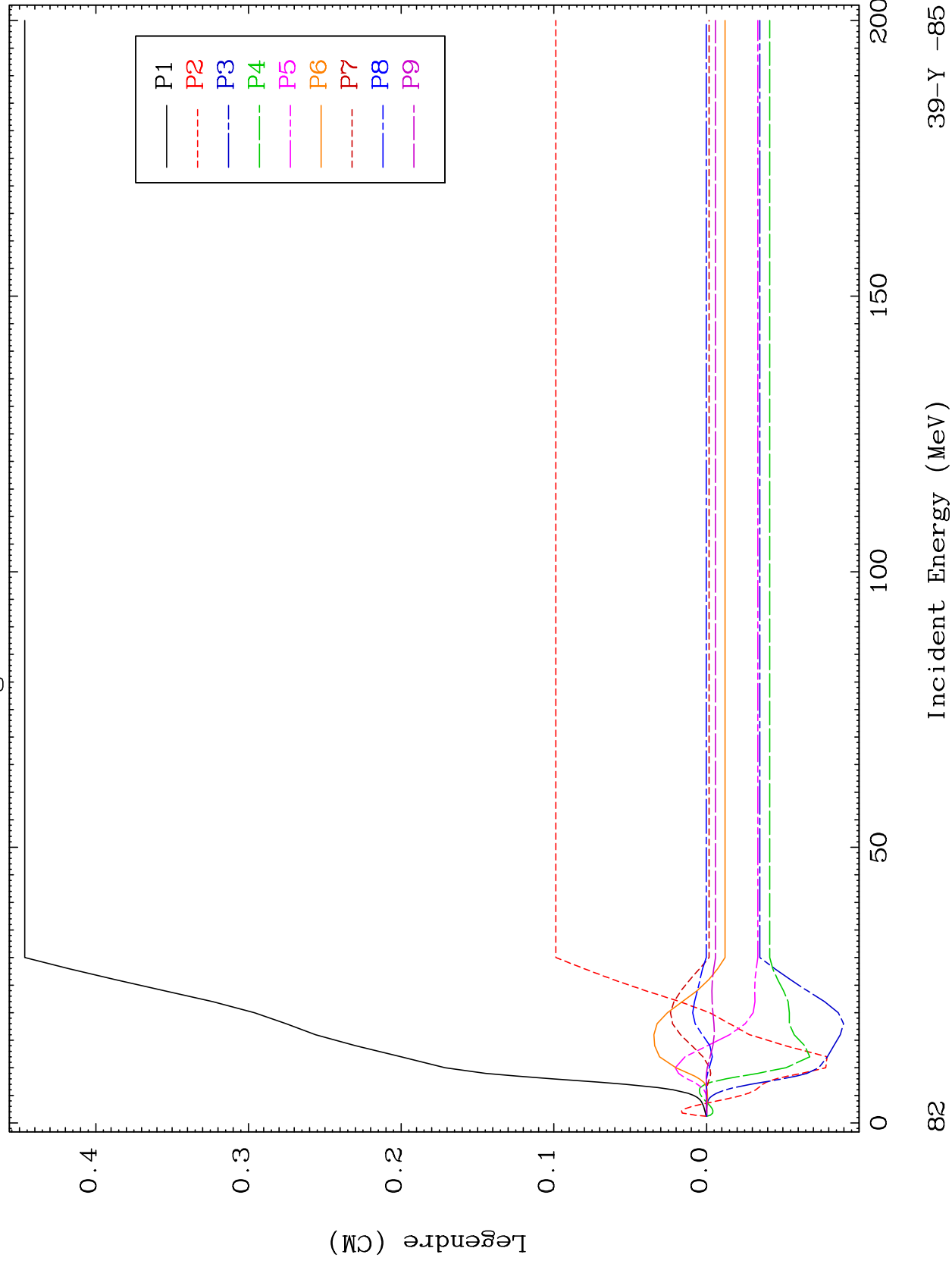


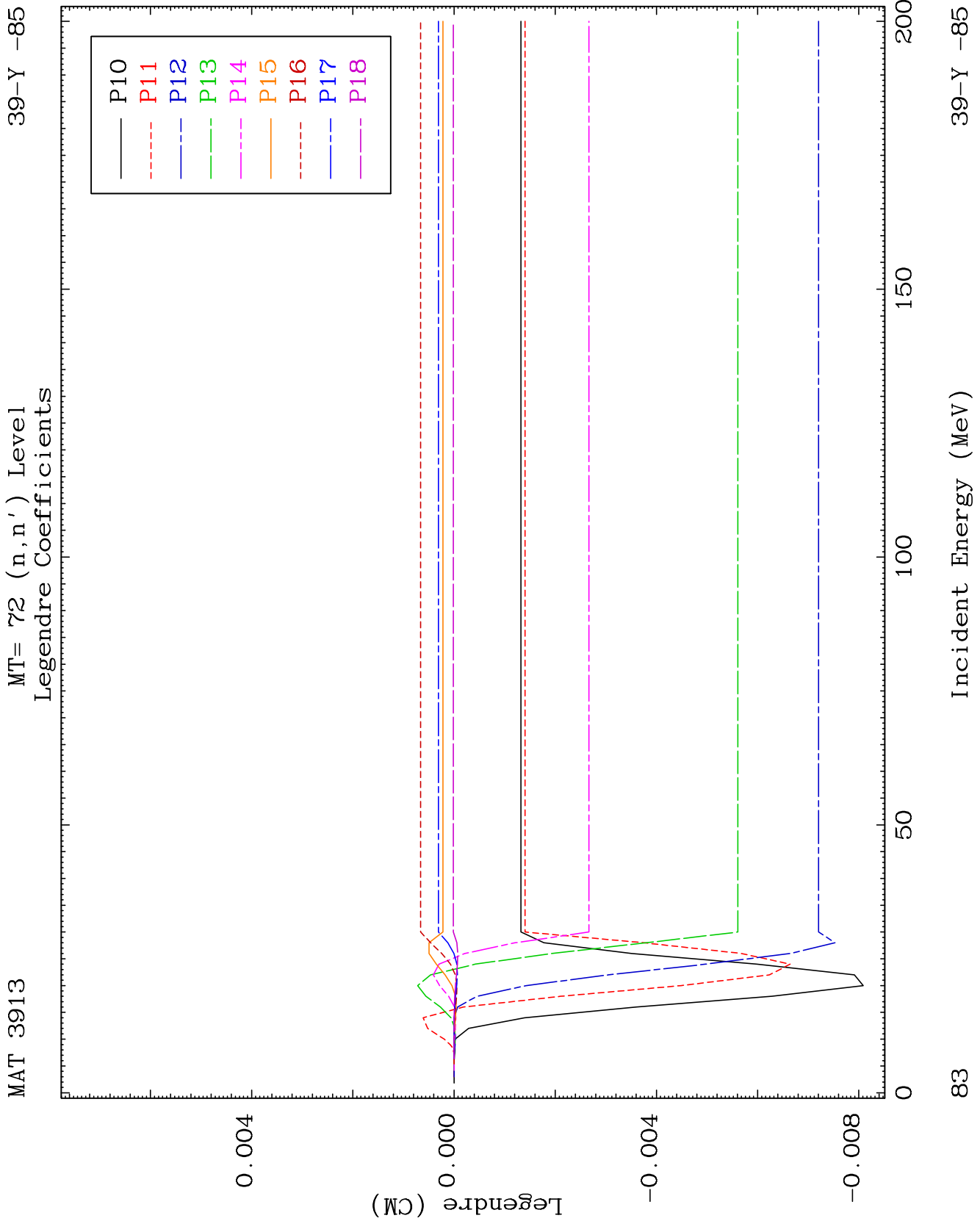


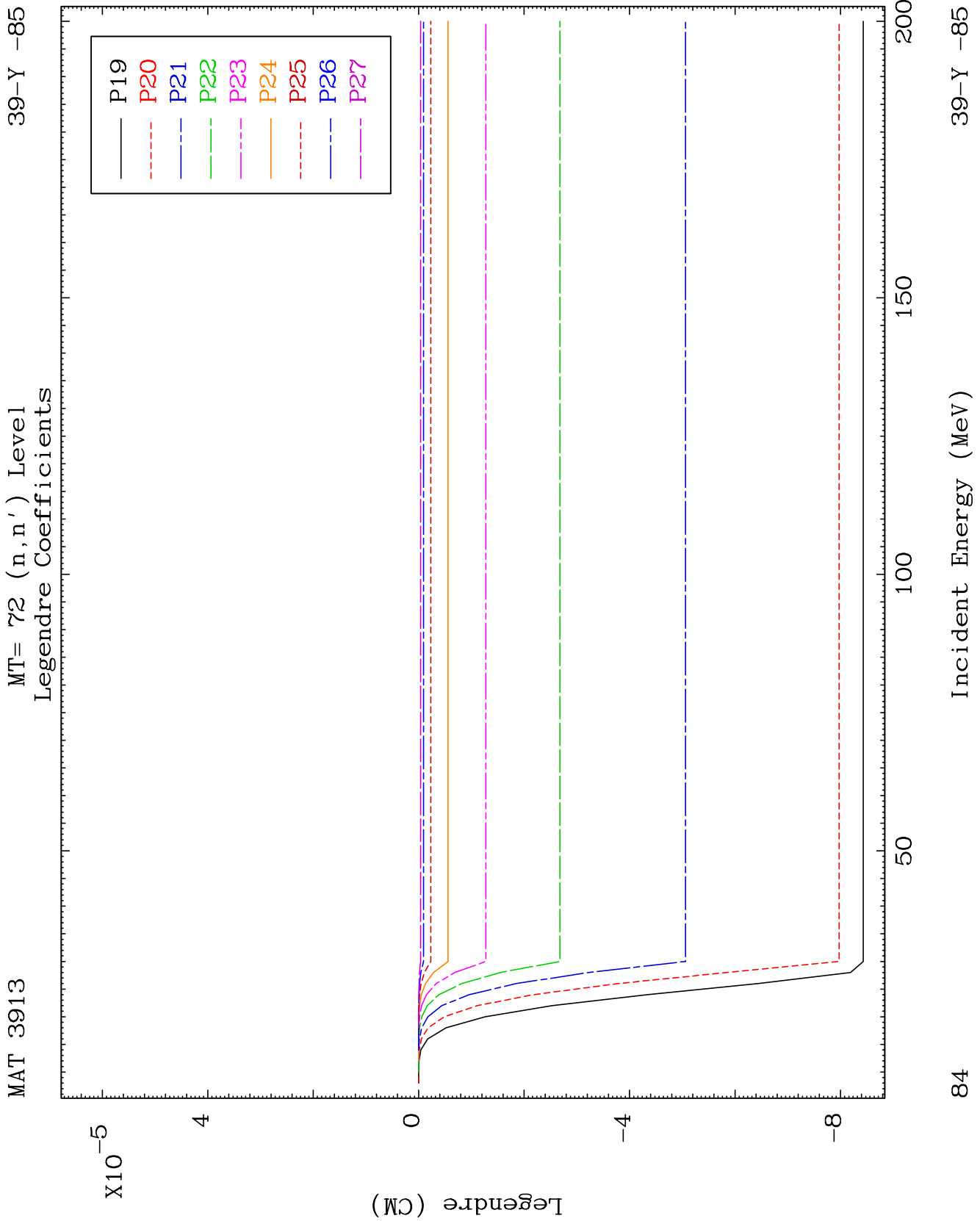
MAT 3913

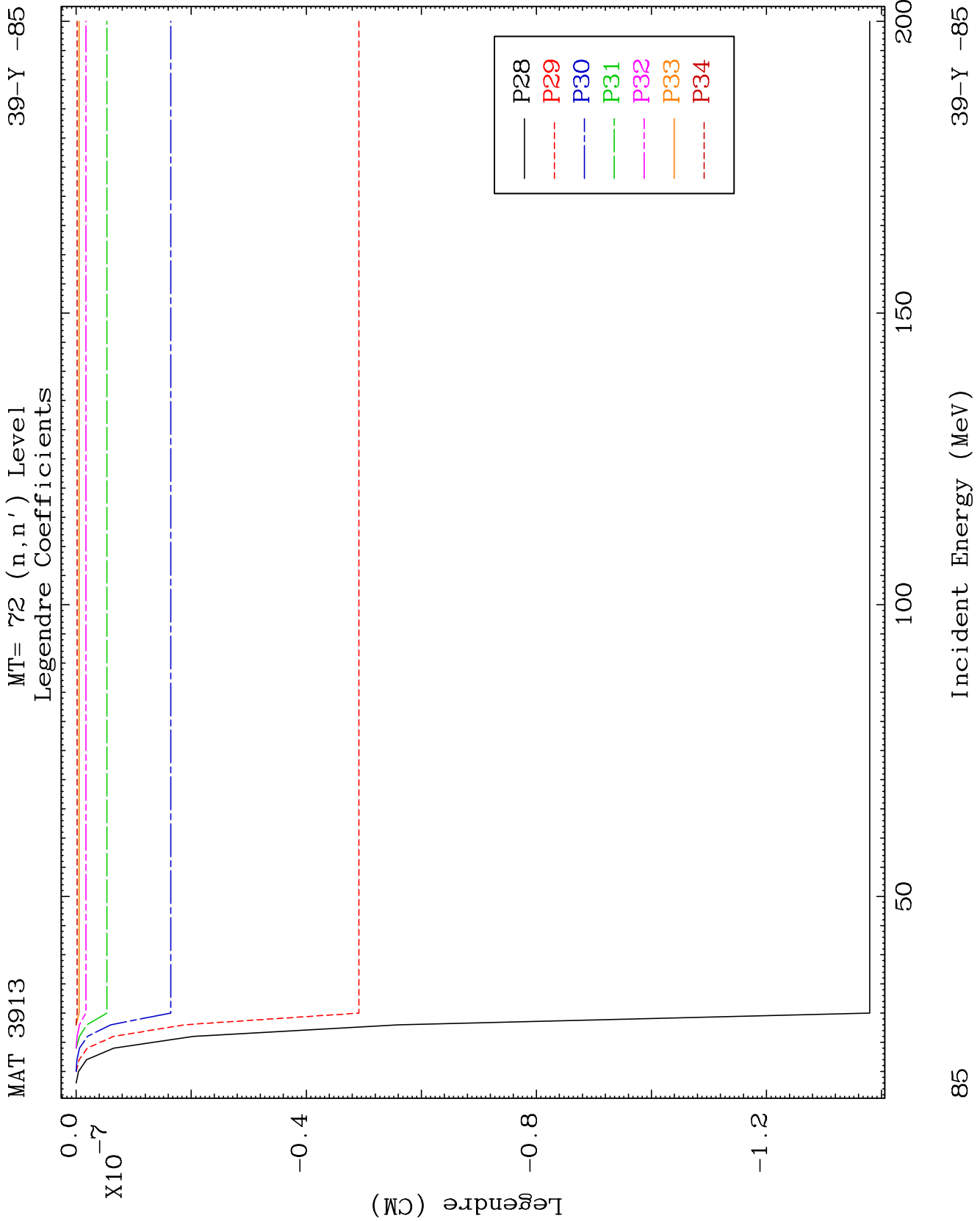
MT= 72 (n, n') Level
Legendre Coefficients

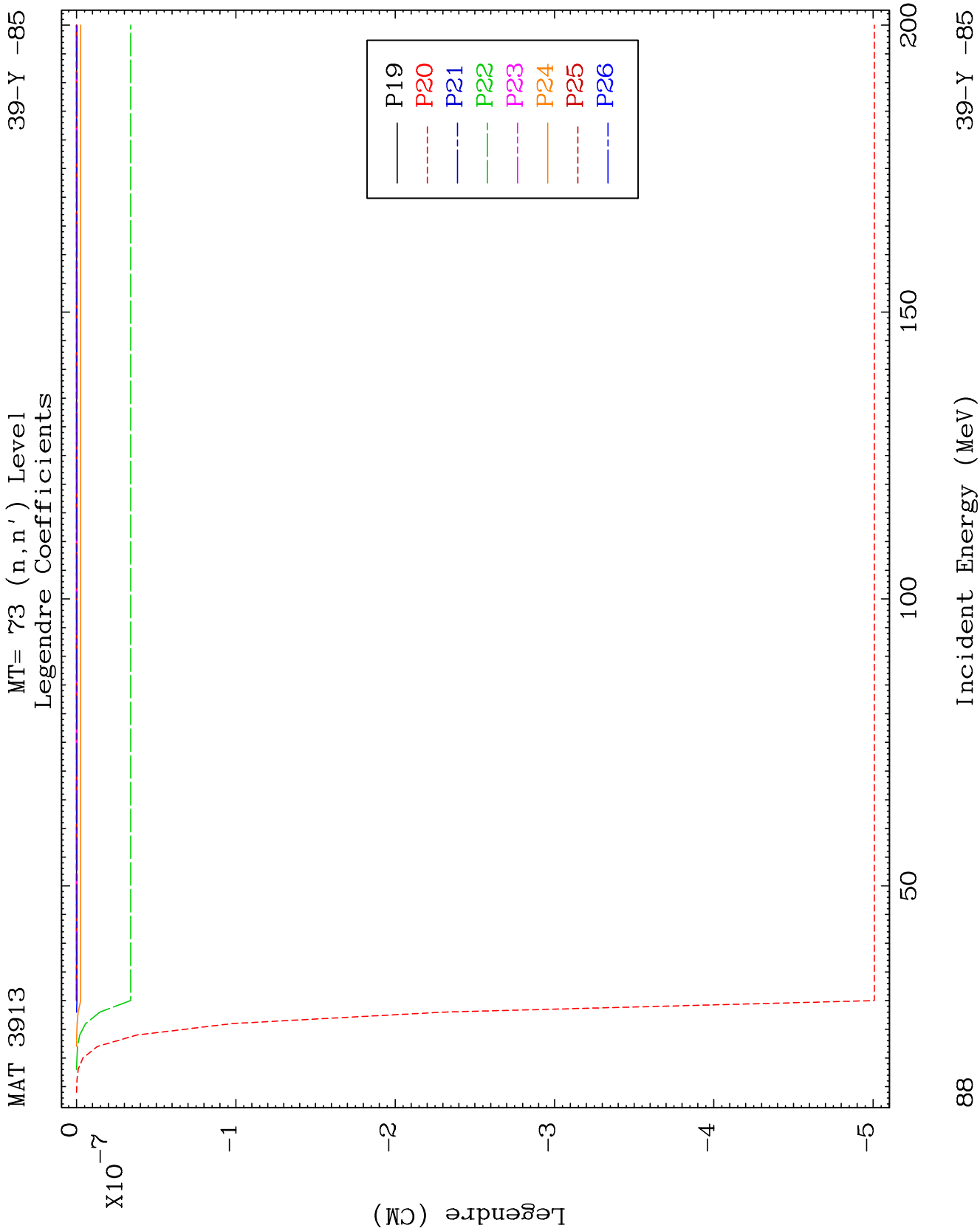
39-Y -85

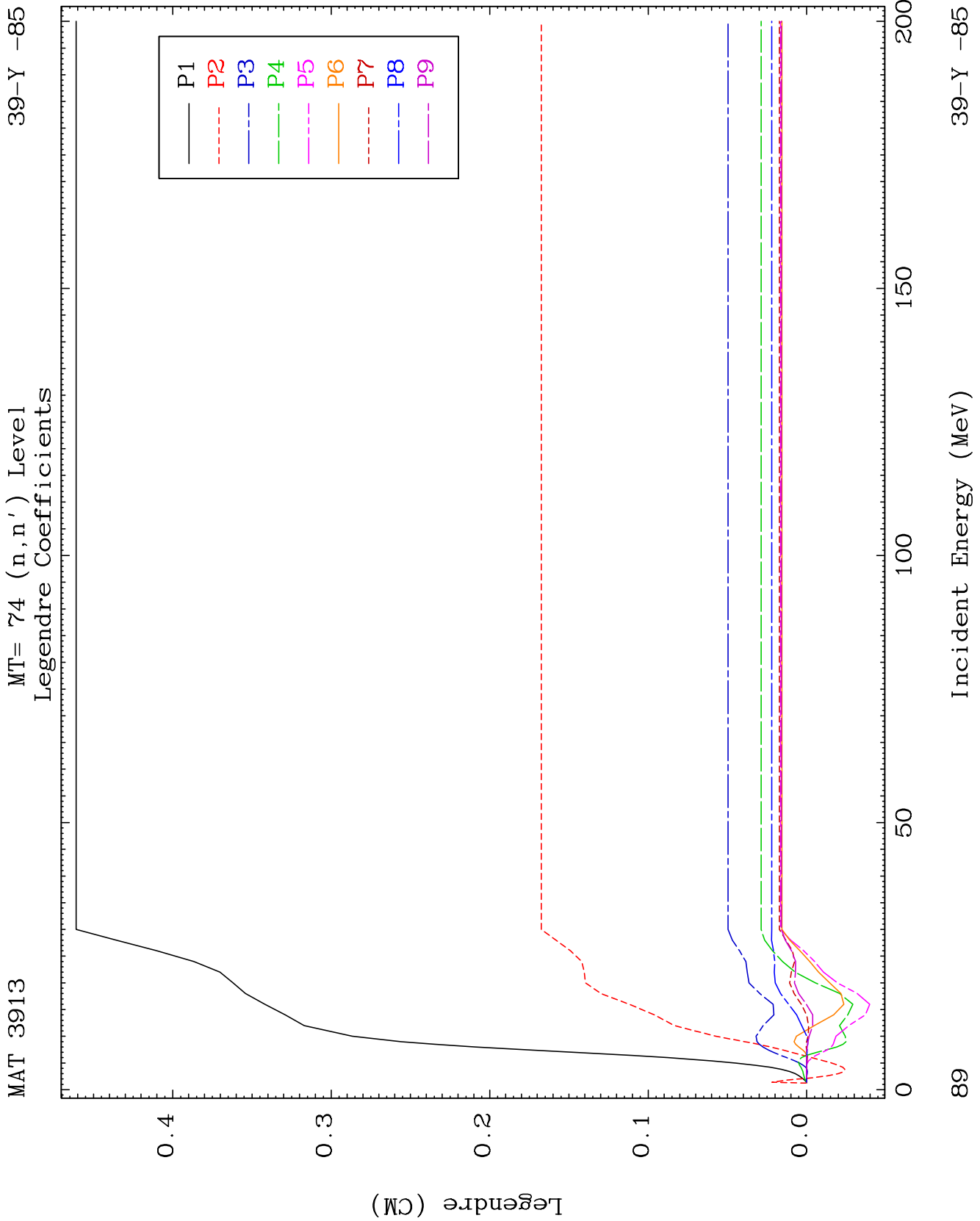


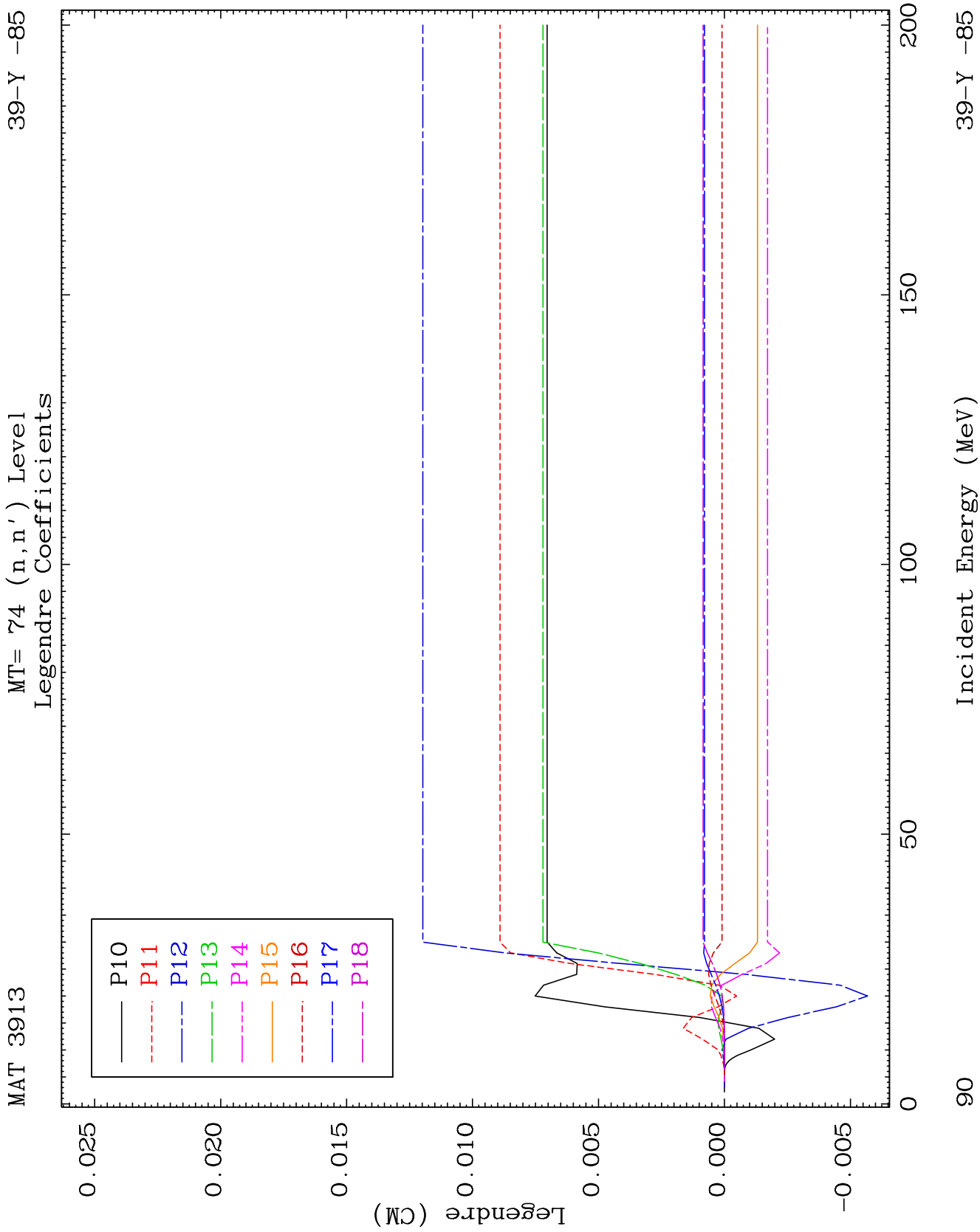


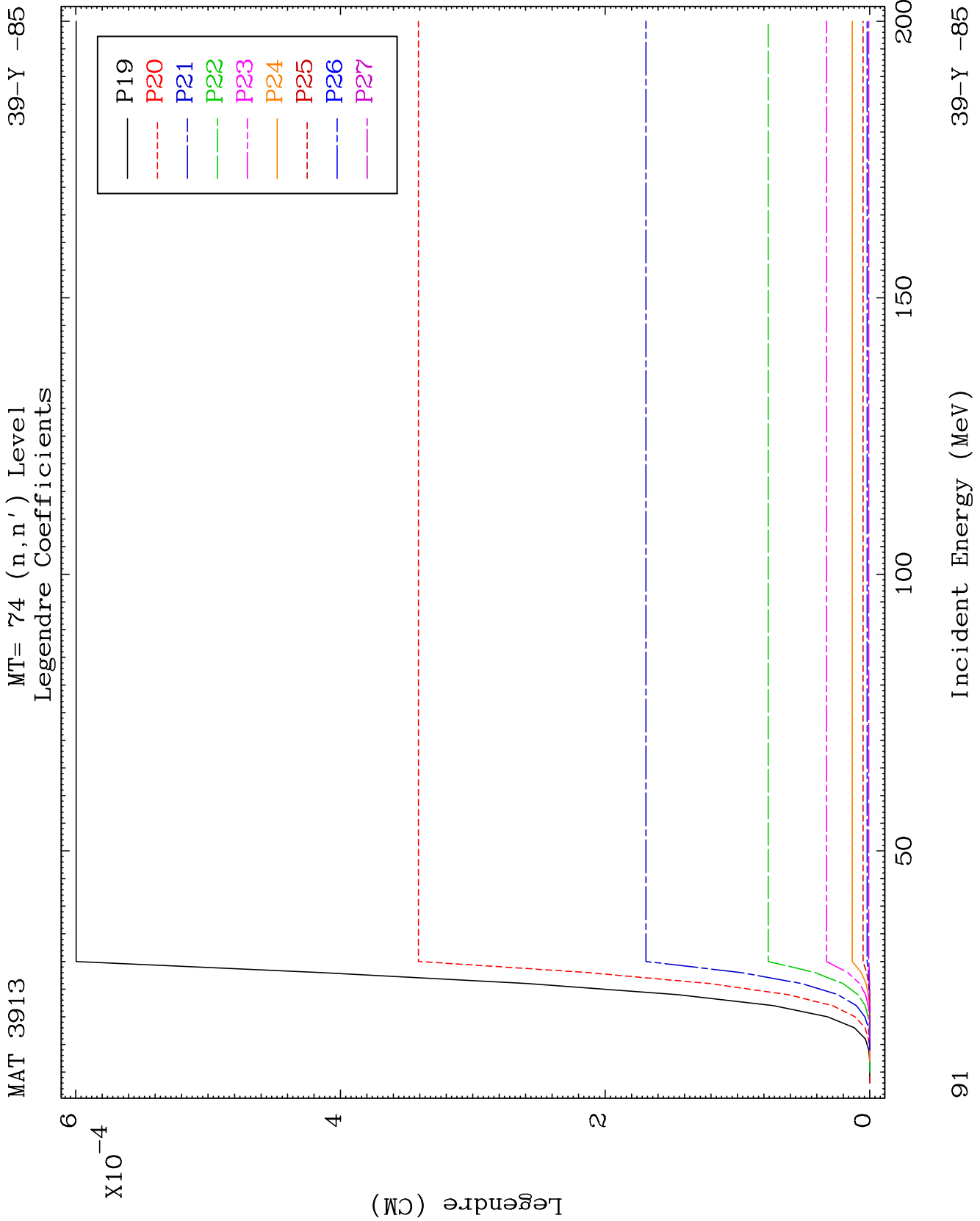


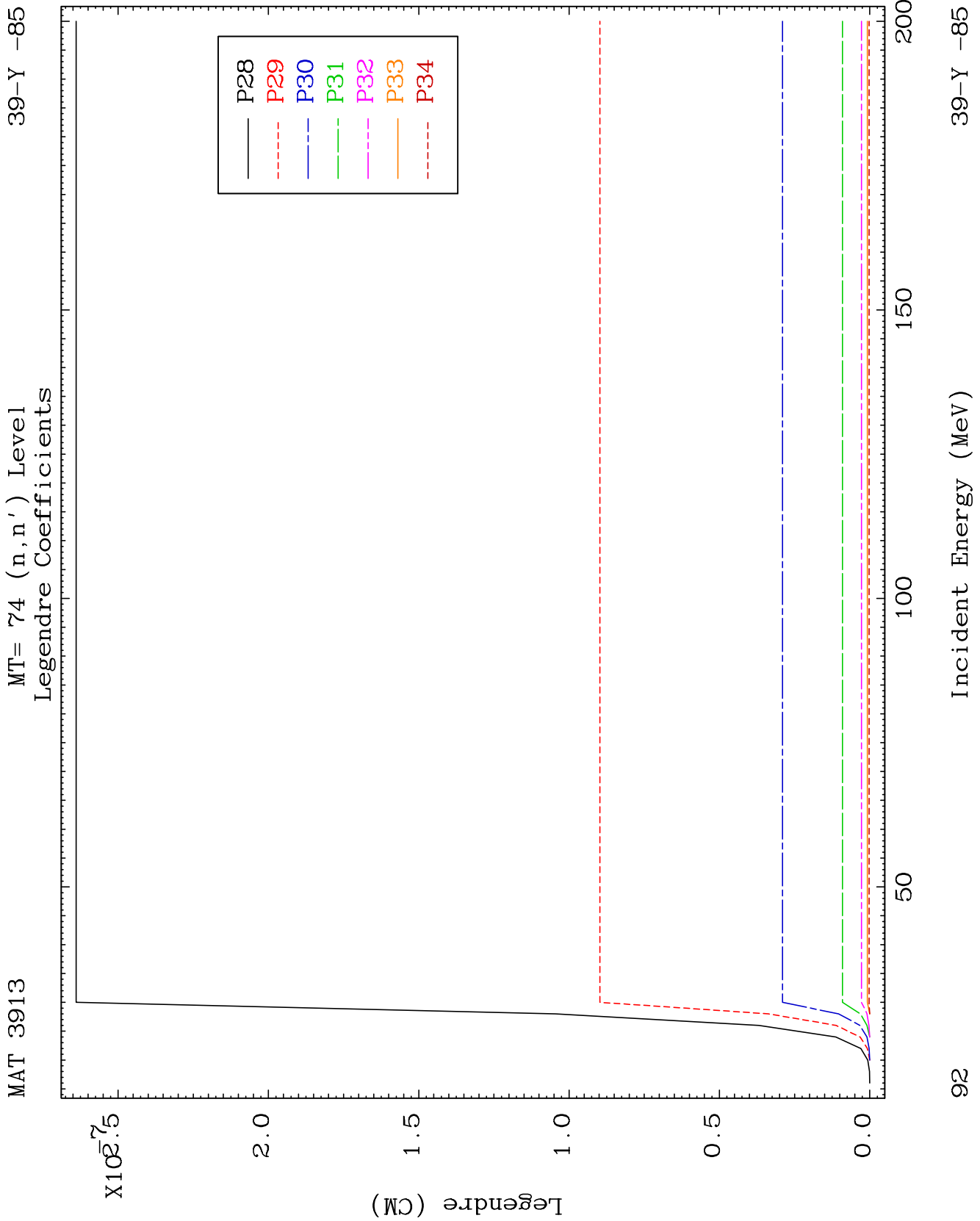


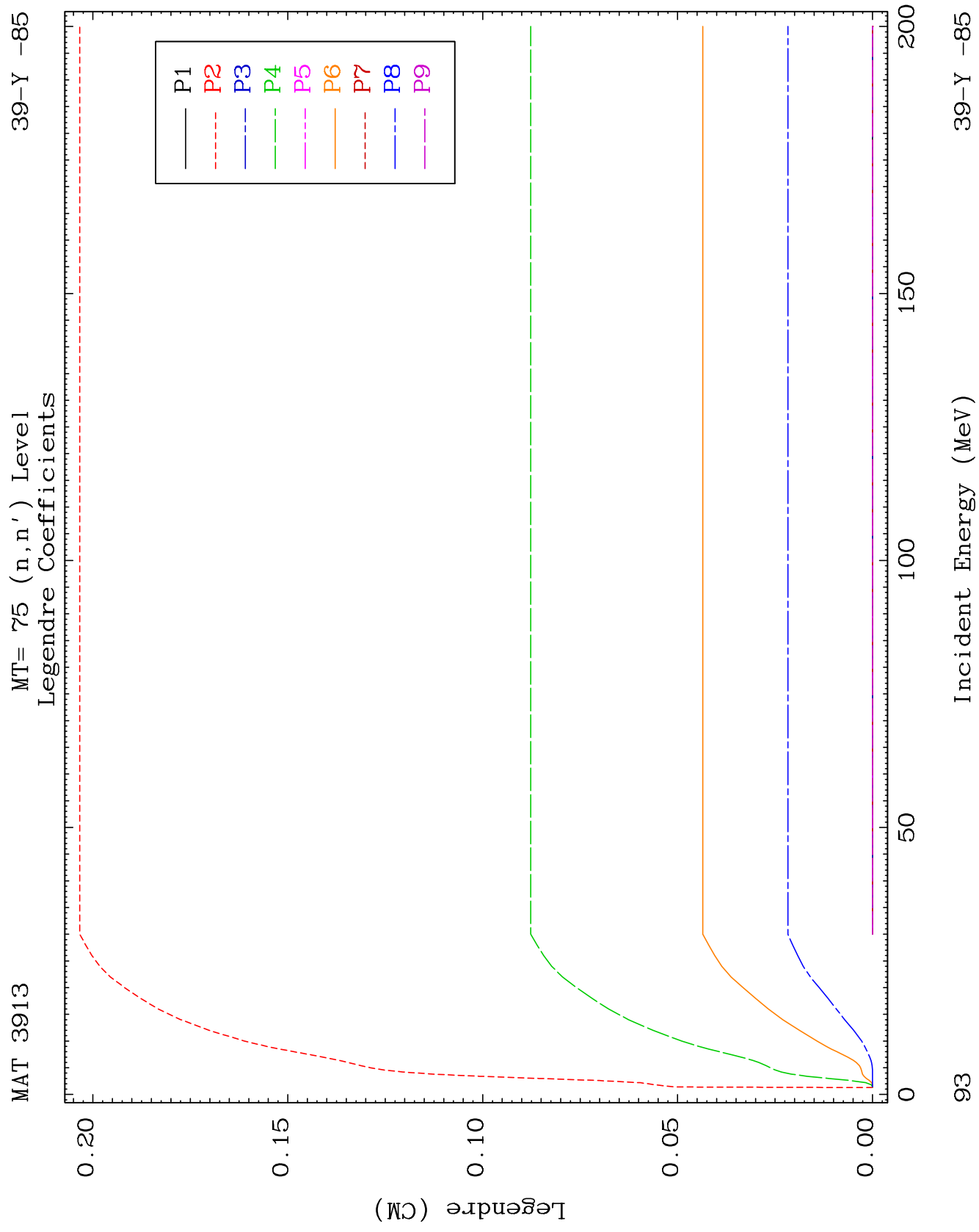


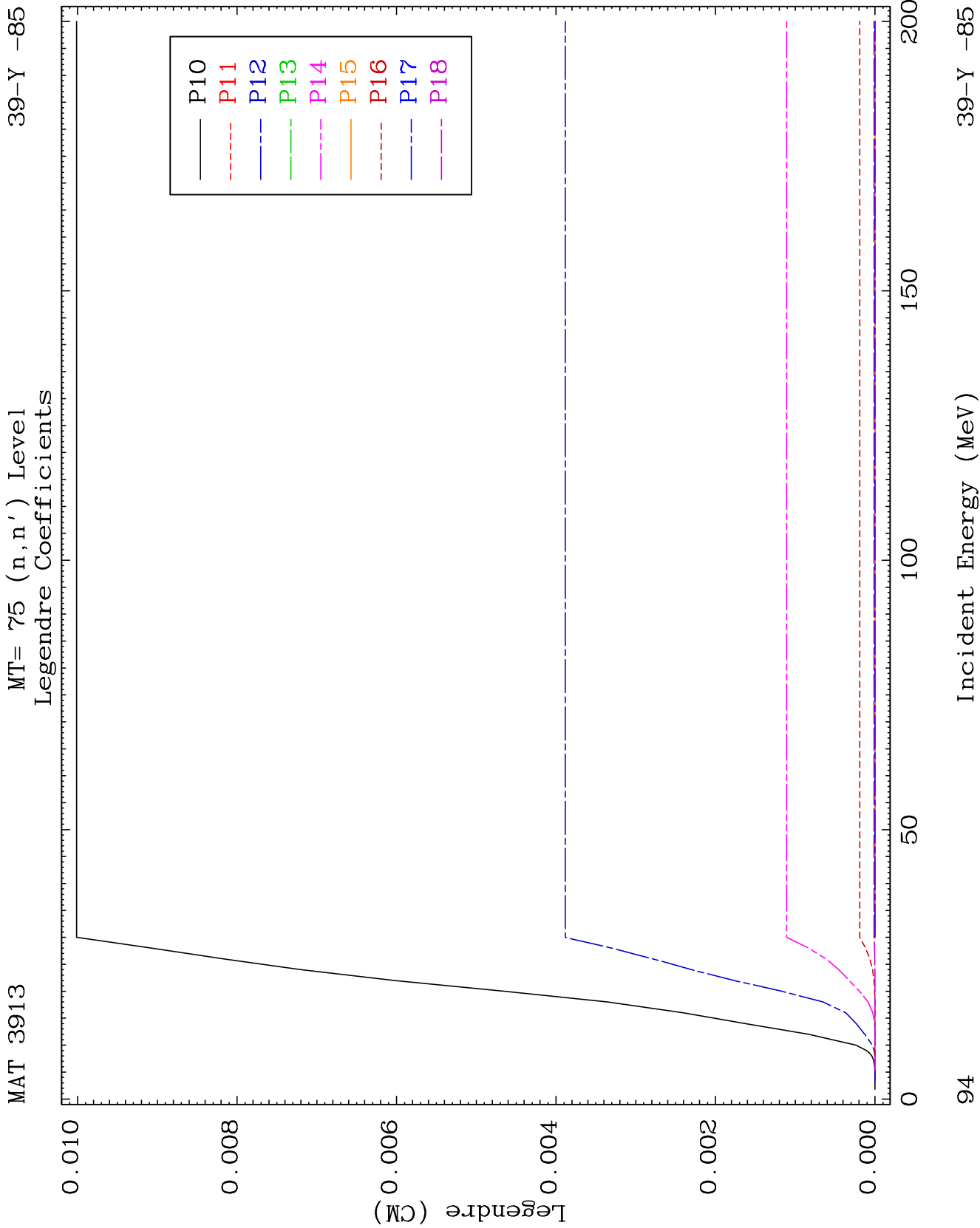


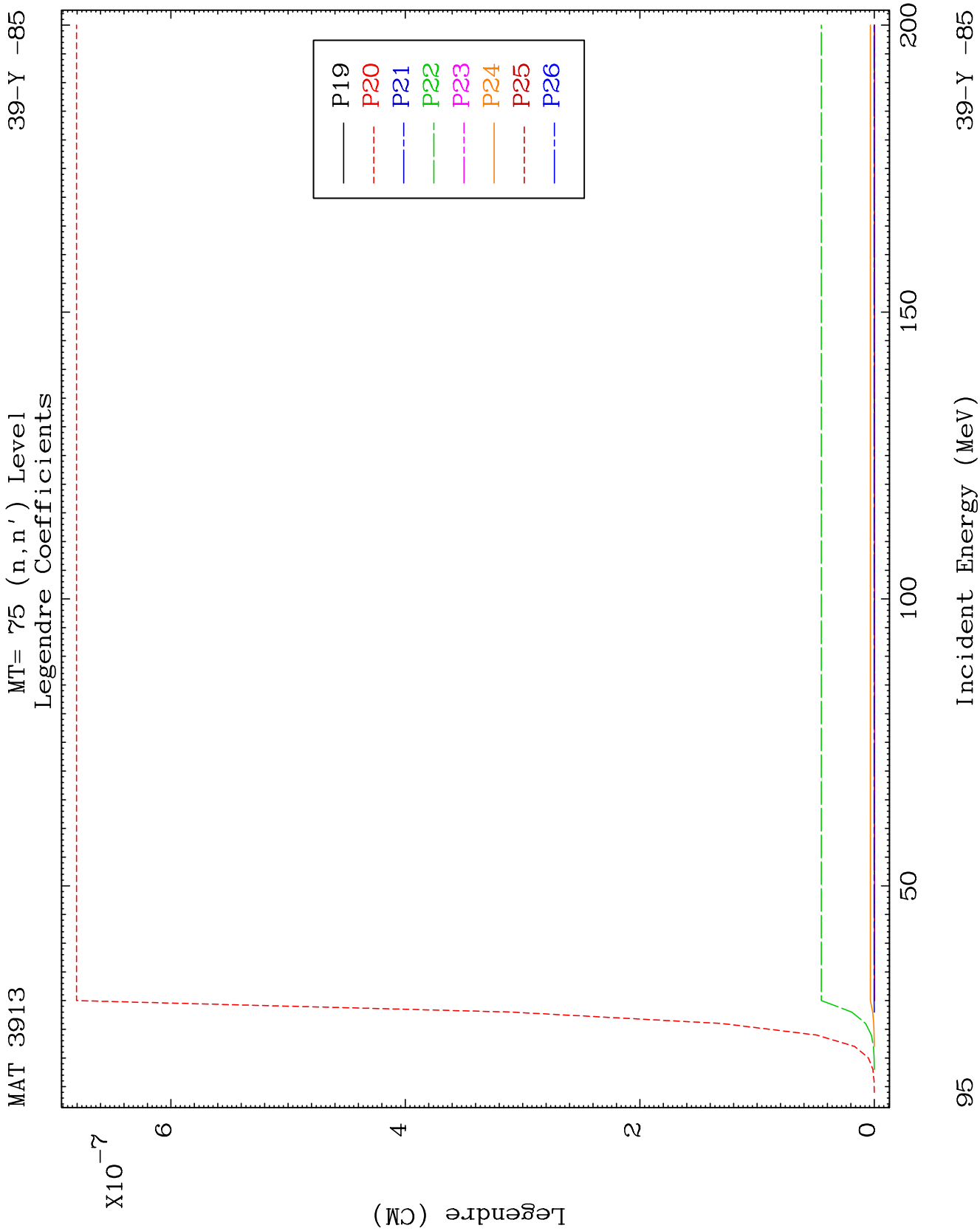








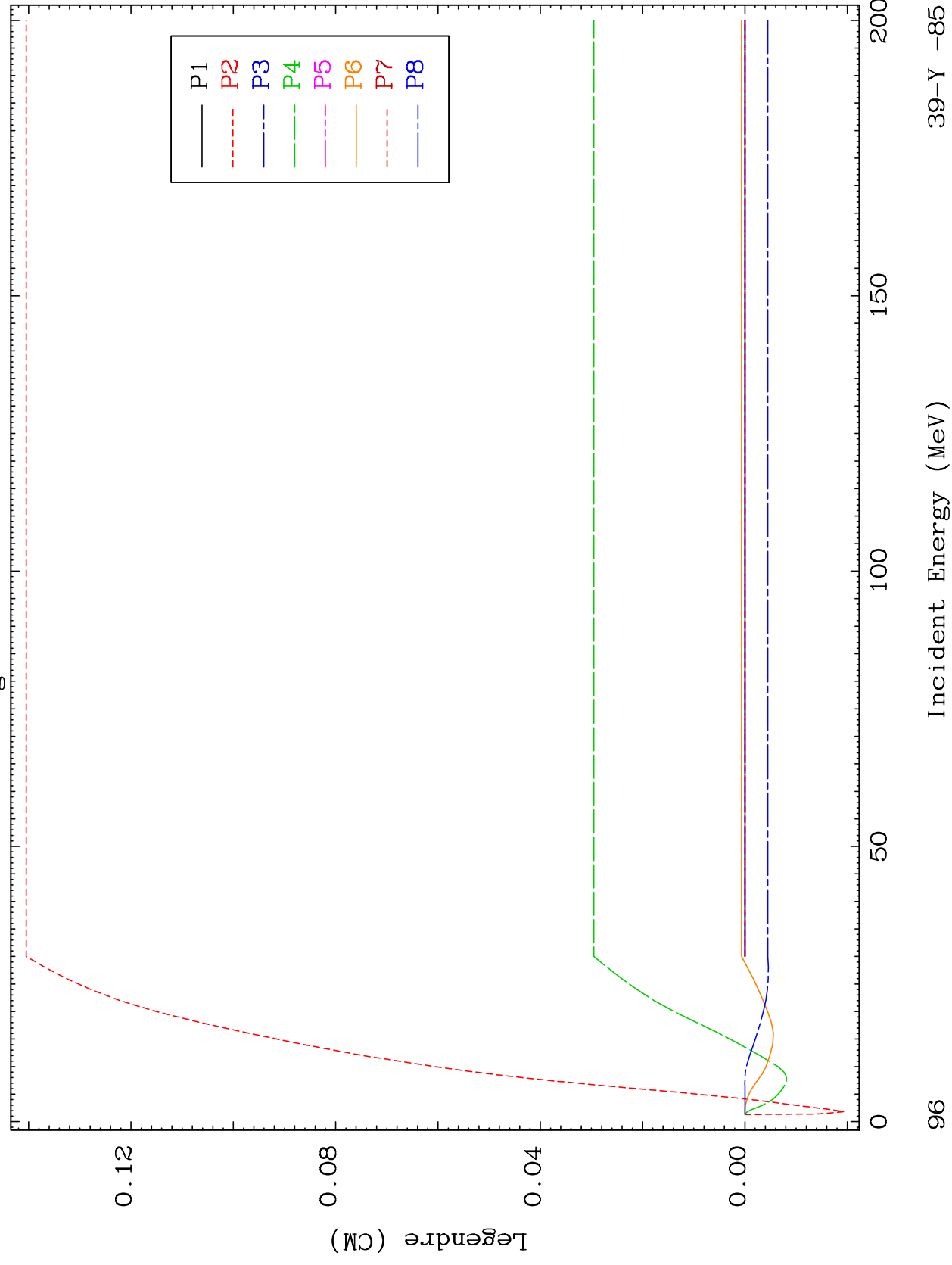


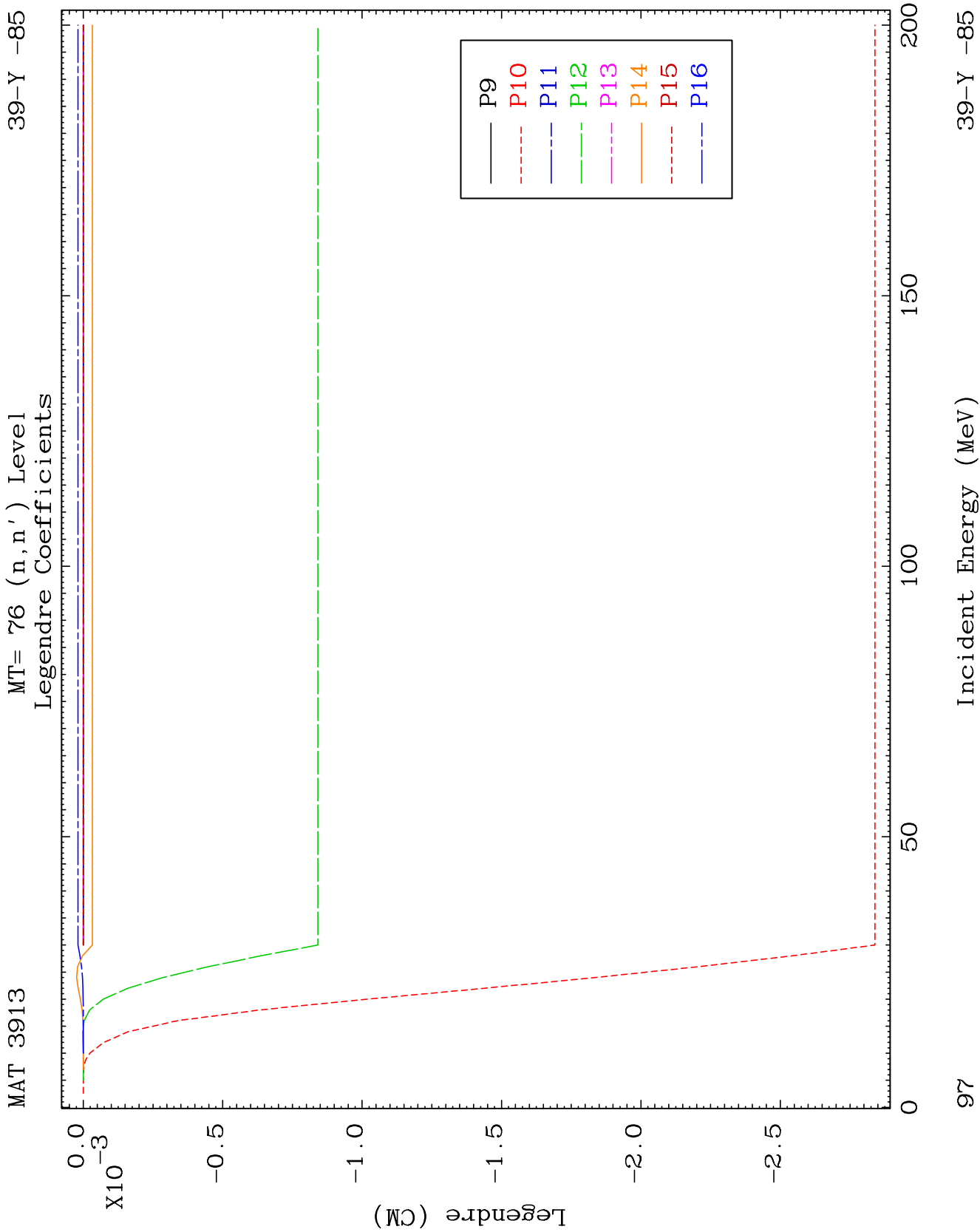


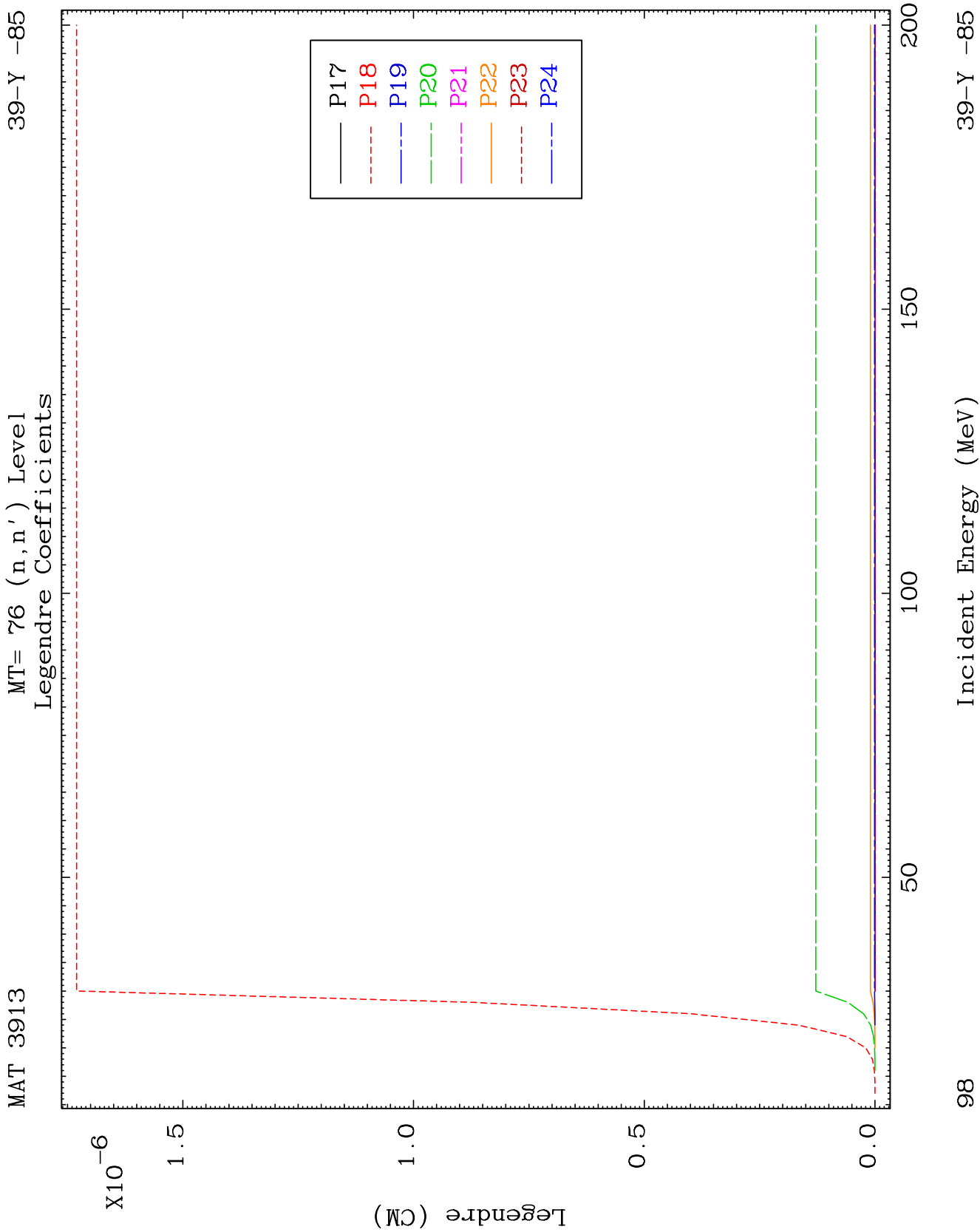
MAT 3913

MT= 76 (n, n') Level
Legendre Coefficients

39-Y -85



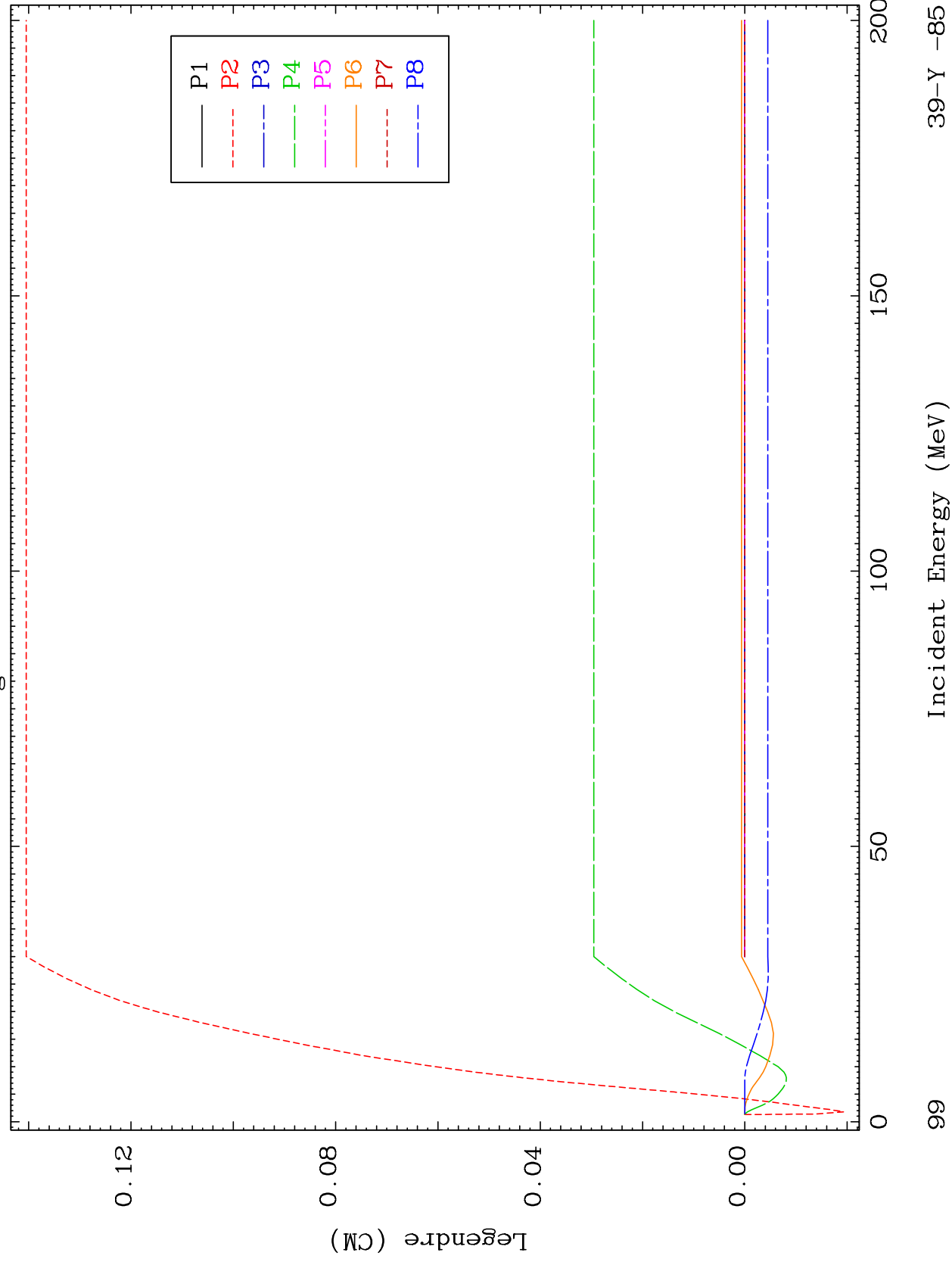


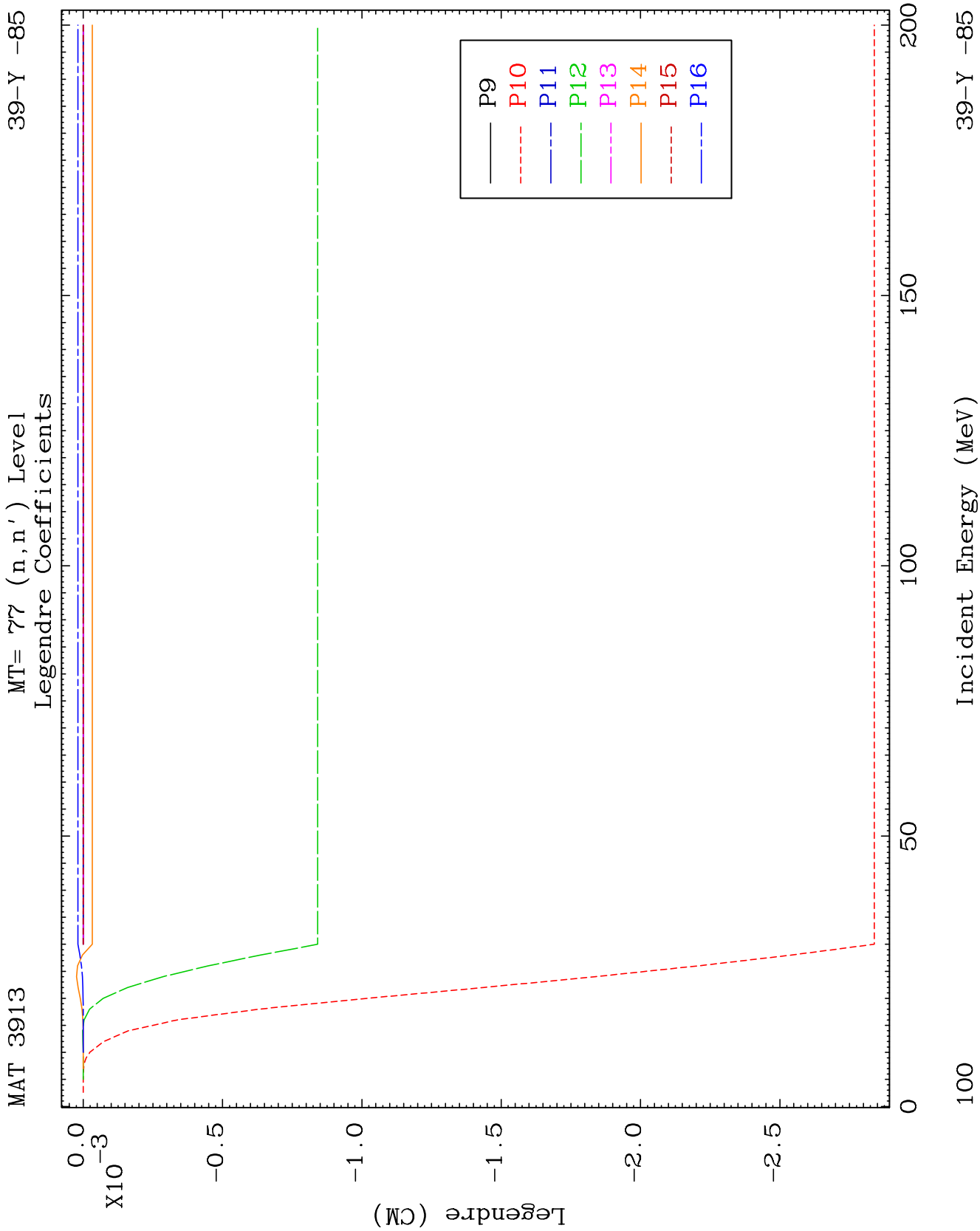


MAT 3913

MT= 77 (n, n') Level
Legendre Coefficients

39-Y -85



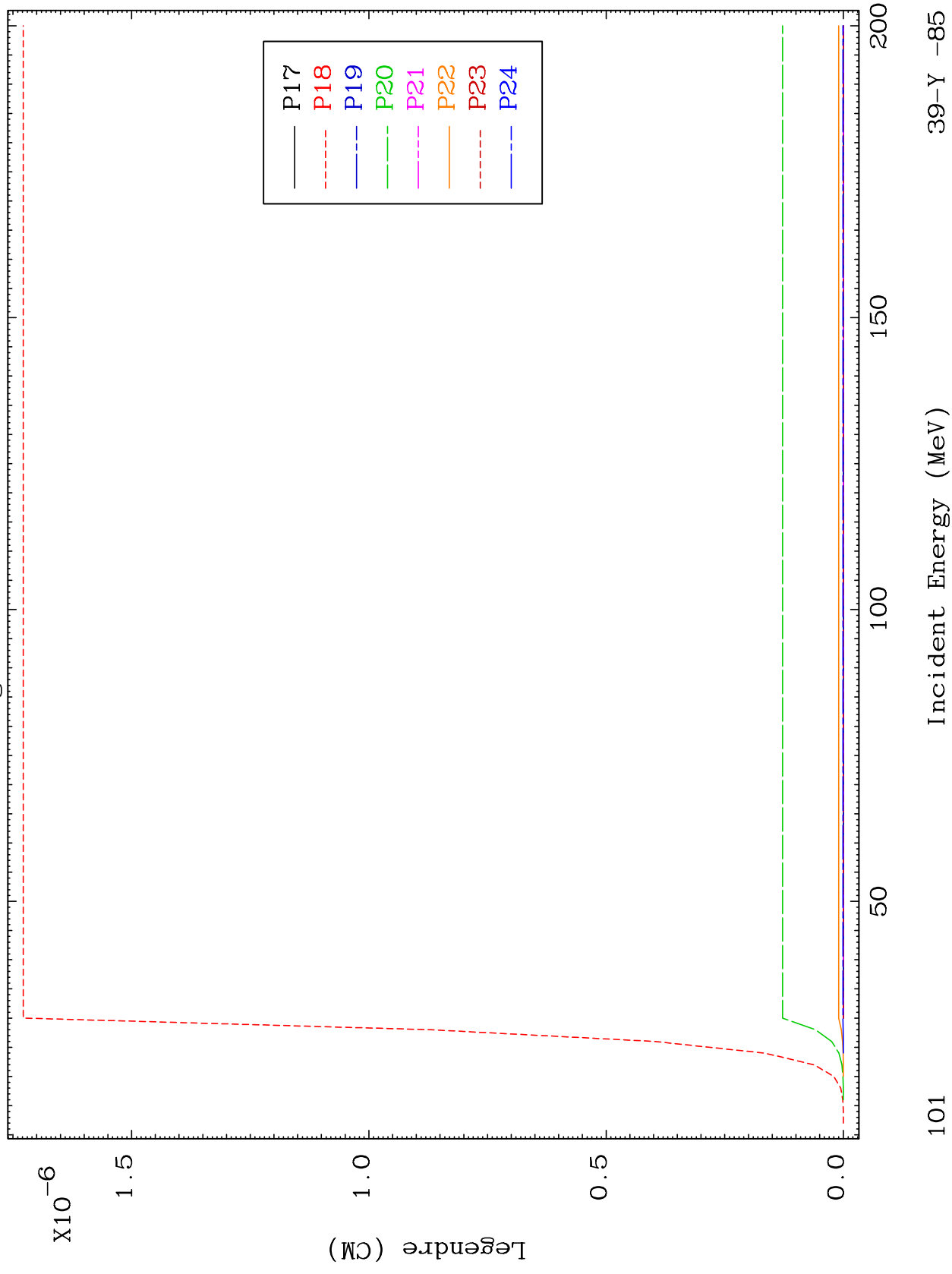


MAT 3913

MT= $\gamma\gamma$ (n, n') Level

39-Y -85

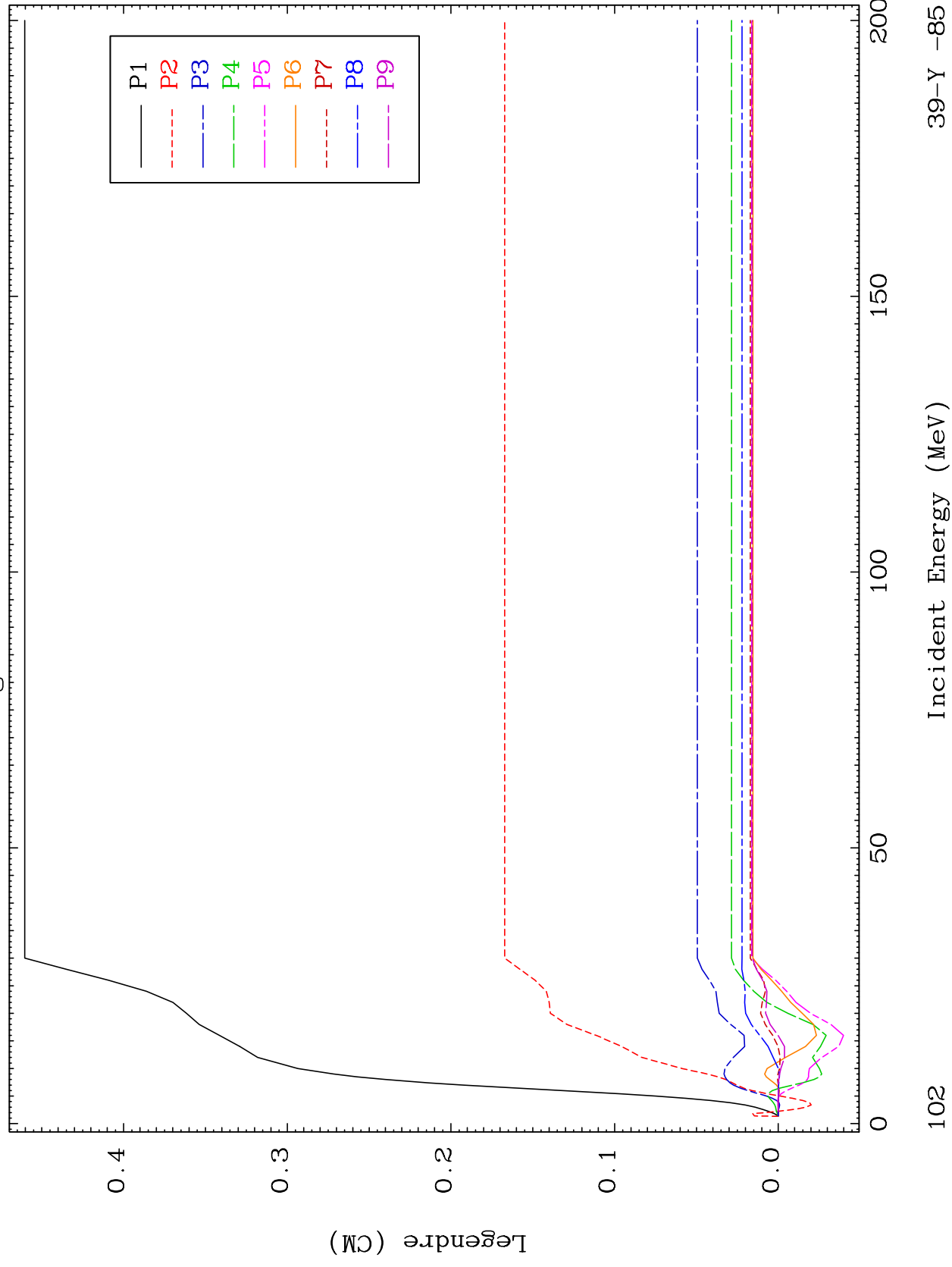
Legendre Coefficients

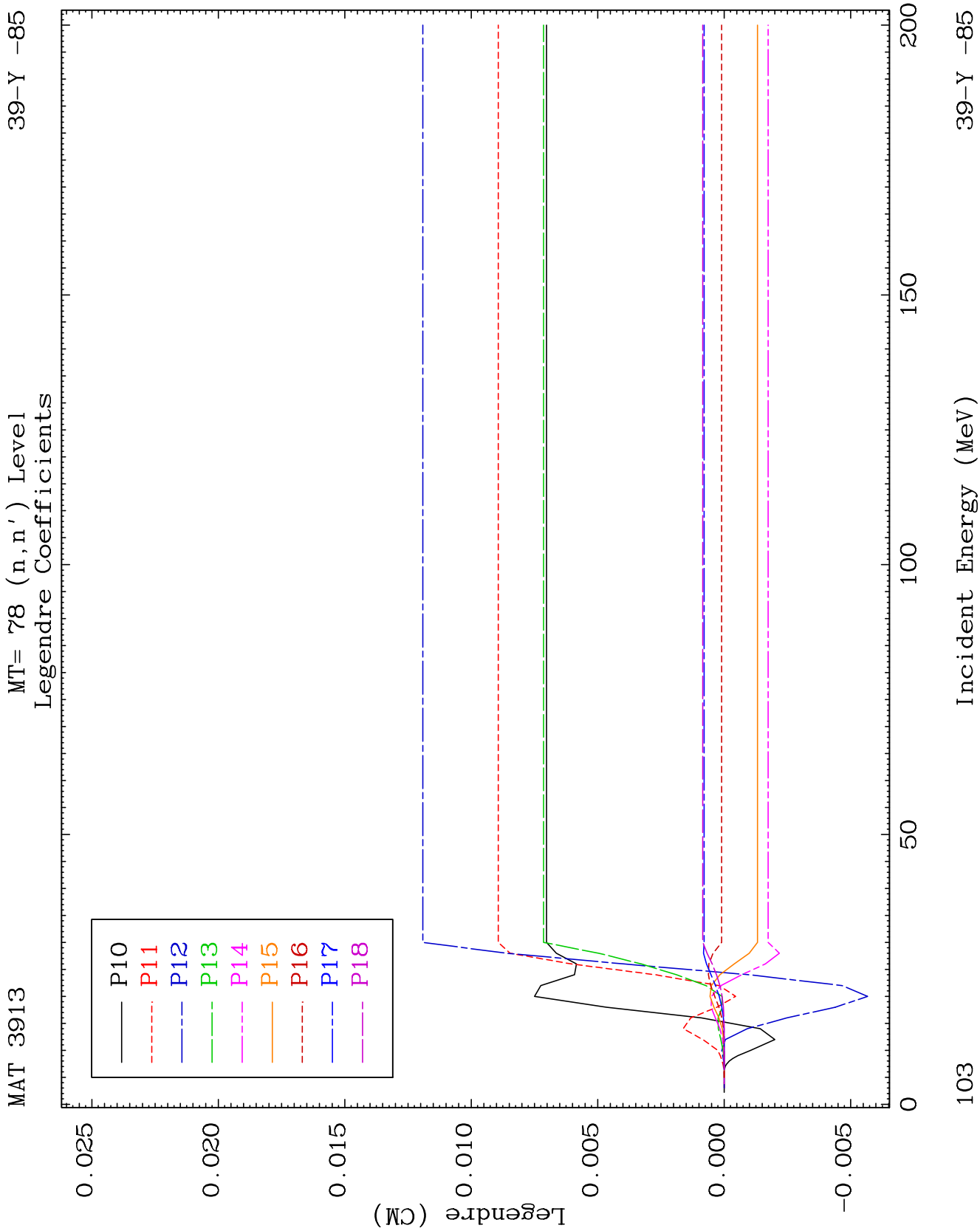


MAT 3913

MT= 78 (n, n') Level
Legendre Coefficients

39-Y -85

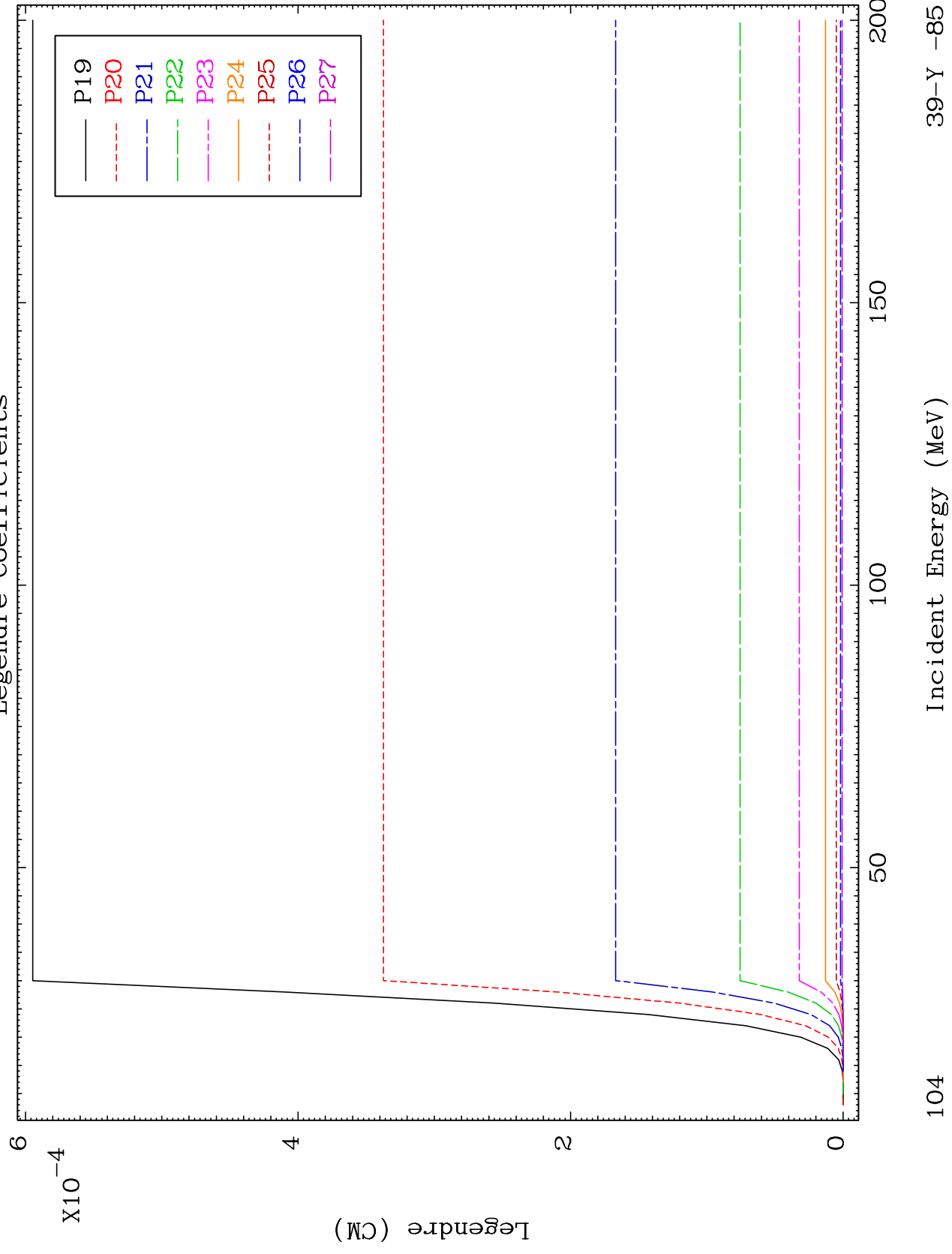


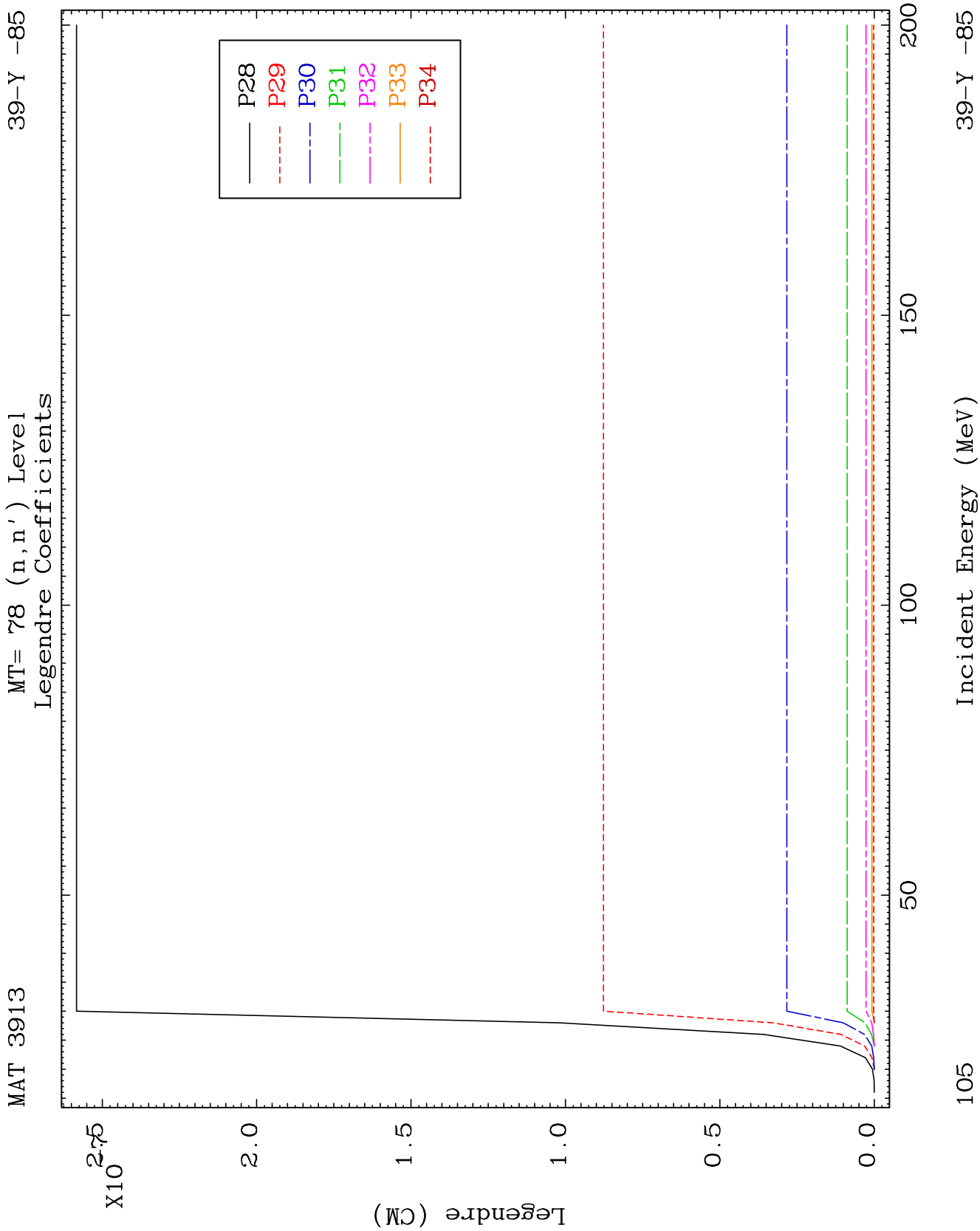


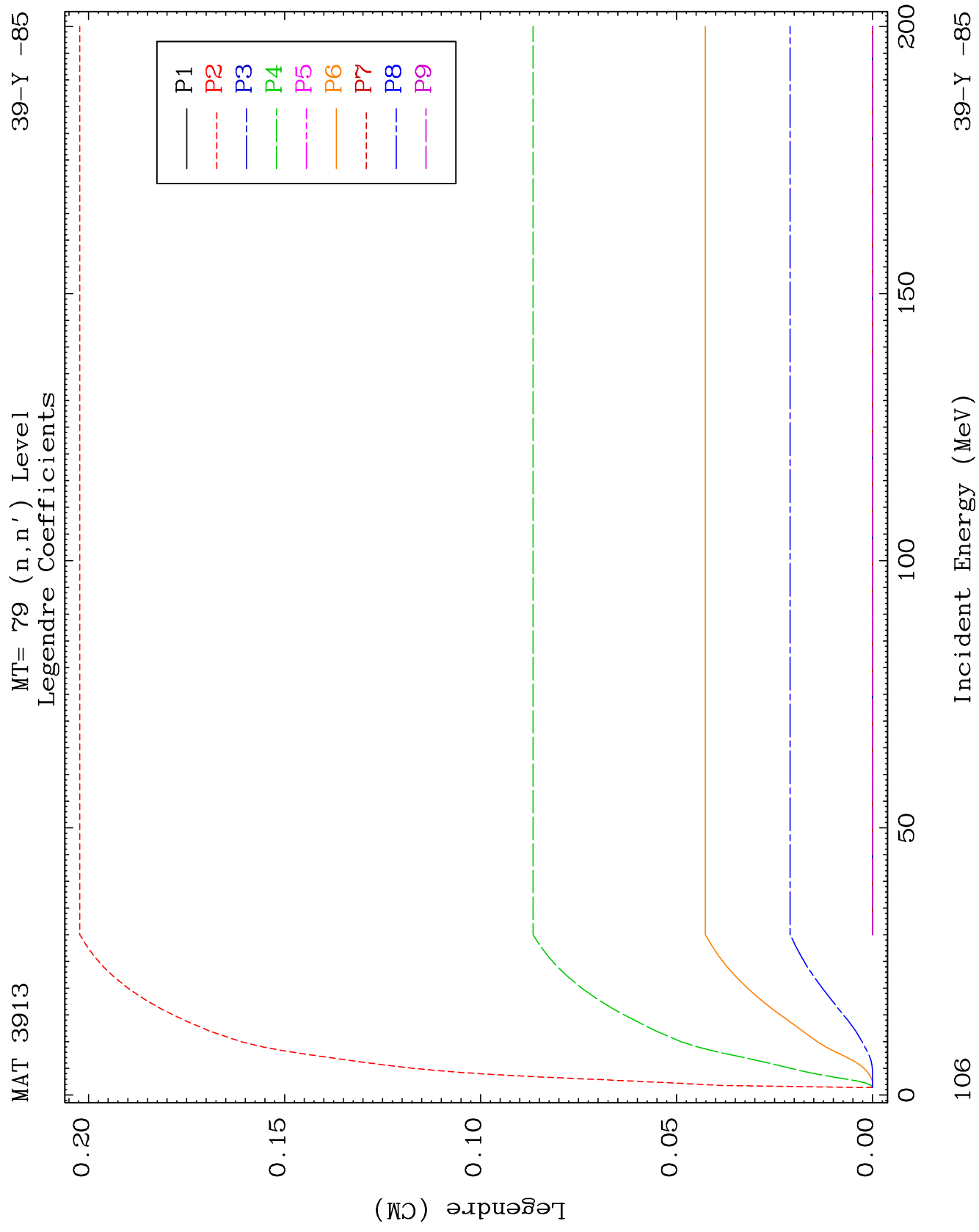
MAT 3913

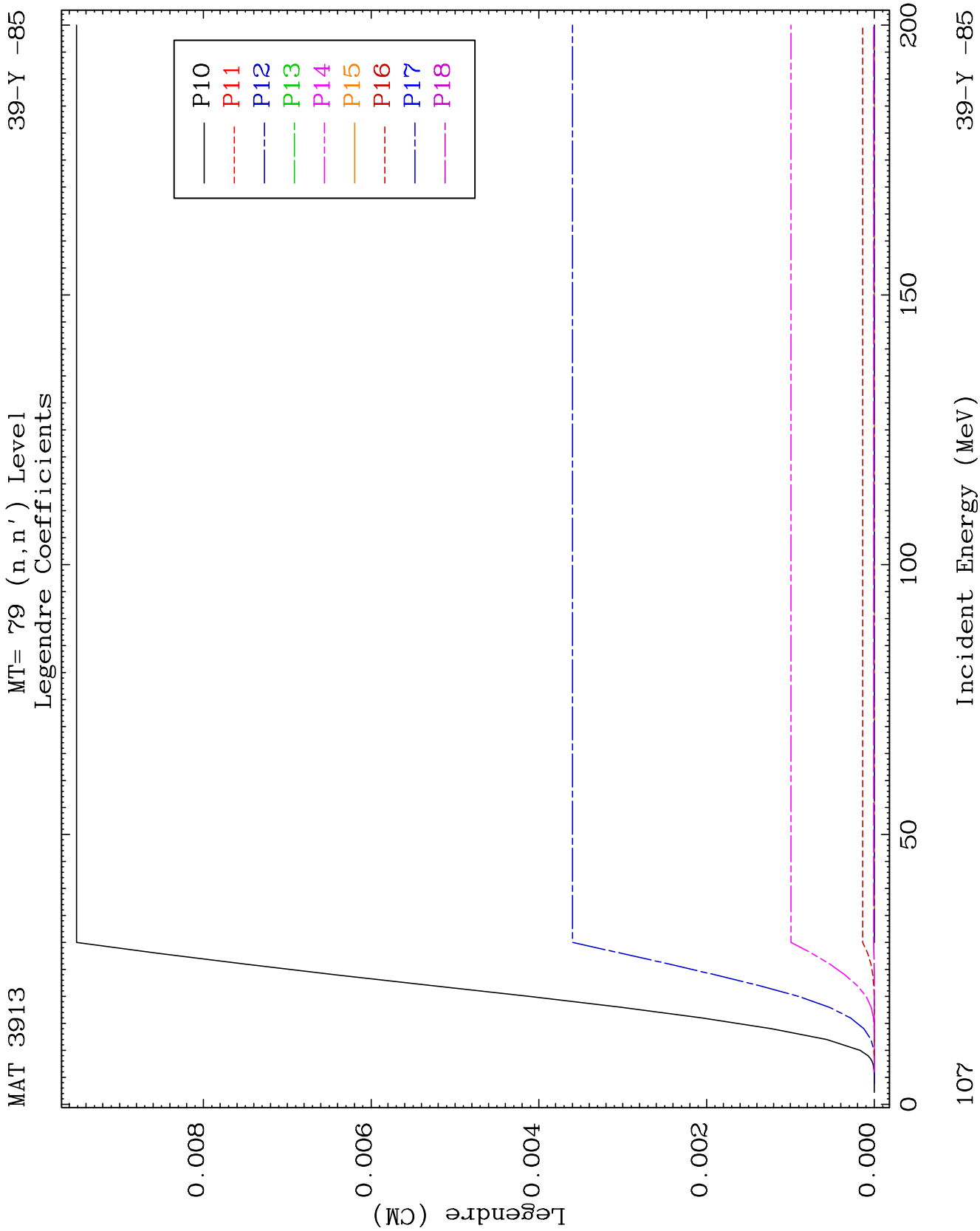
MT= 78 (n, n') Level
Legendre Coefficients

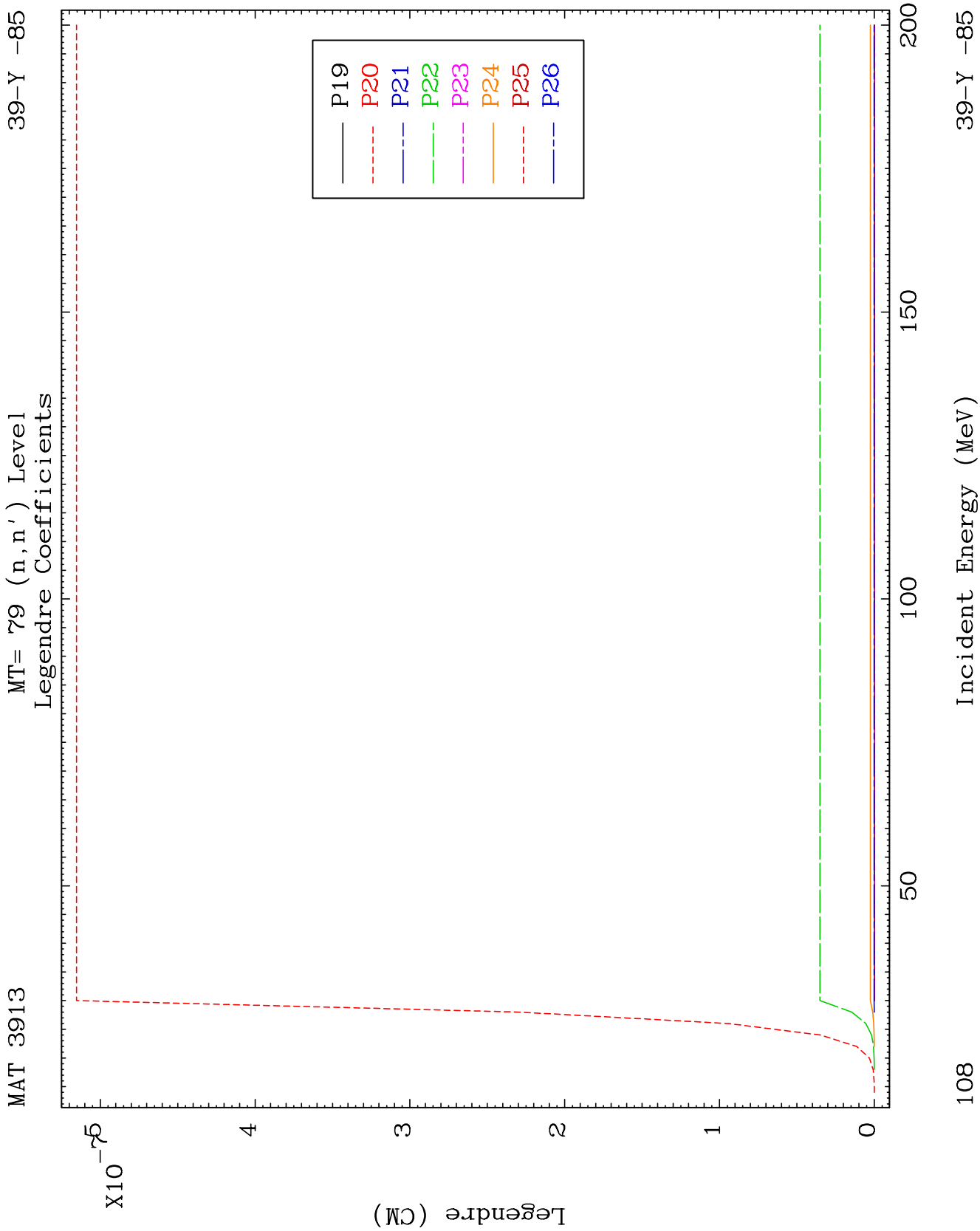
39-Y -85

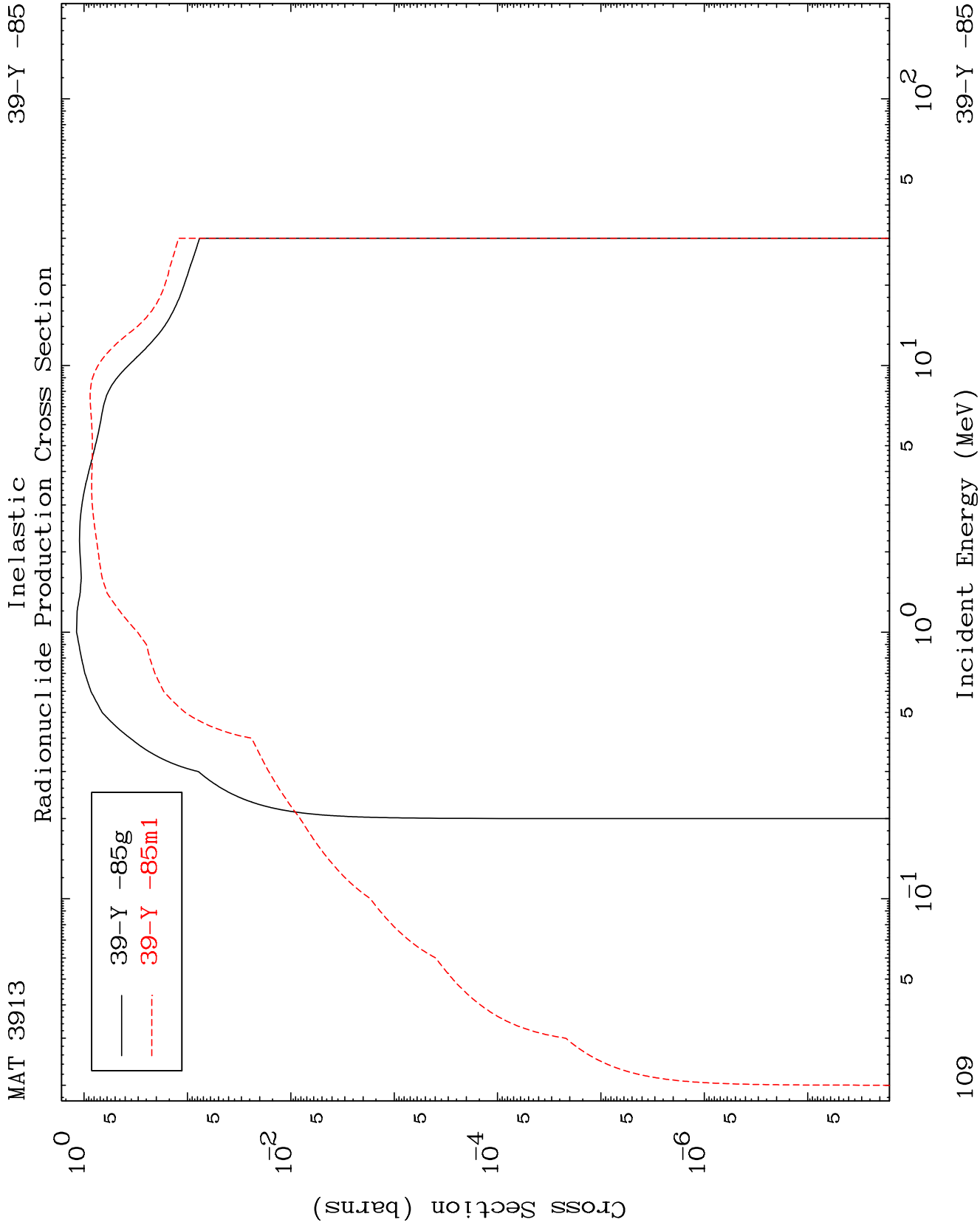








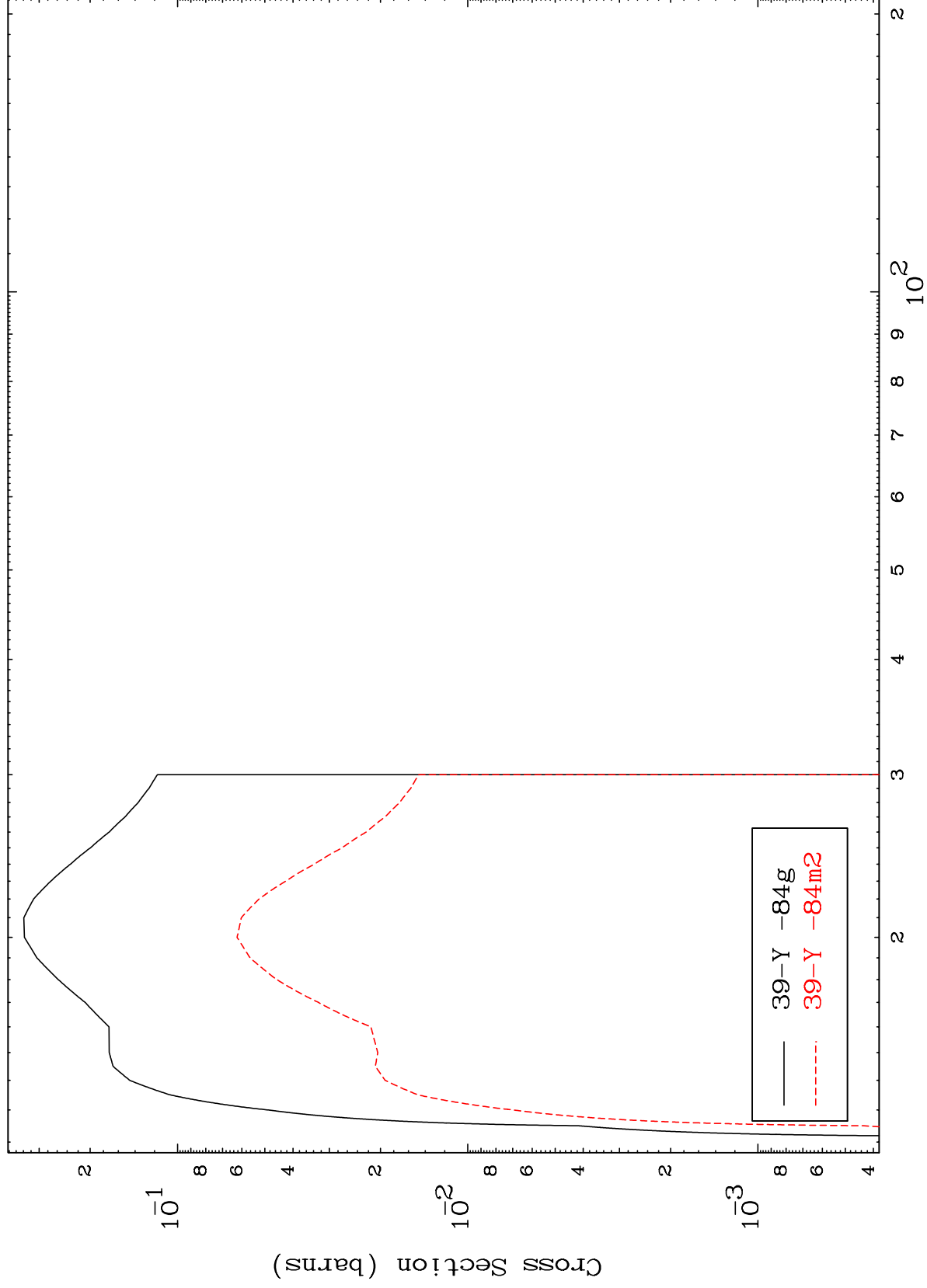




MAT 3913

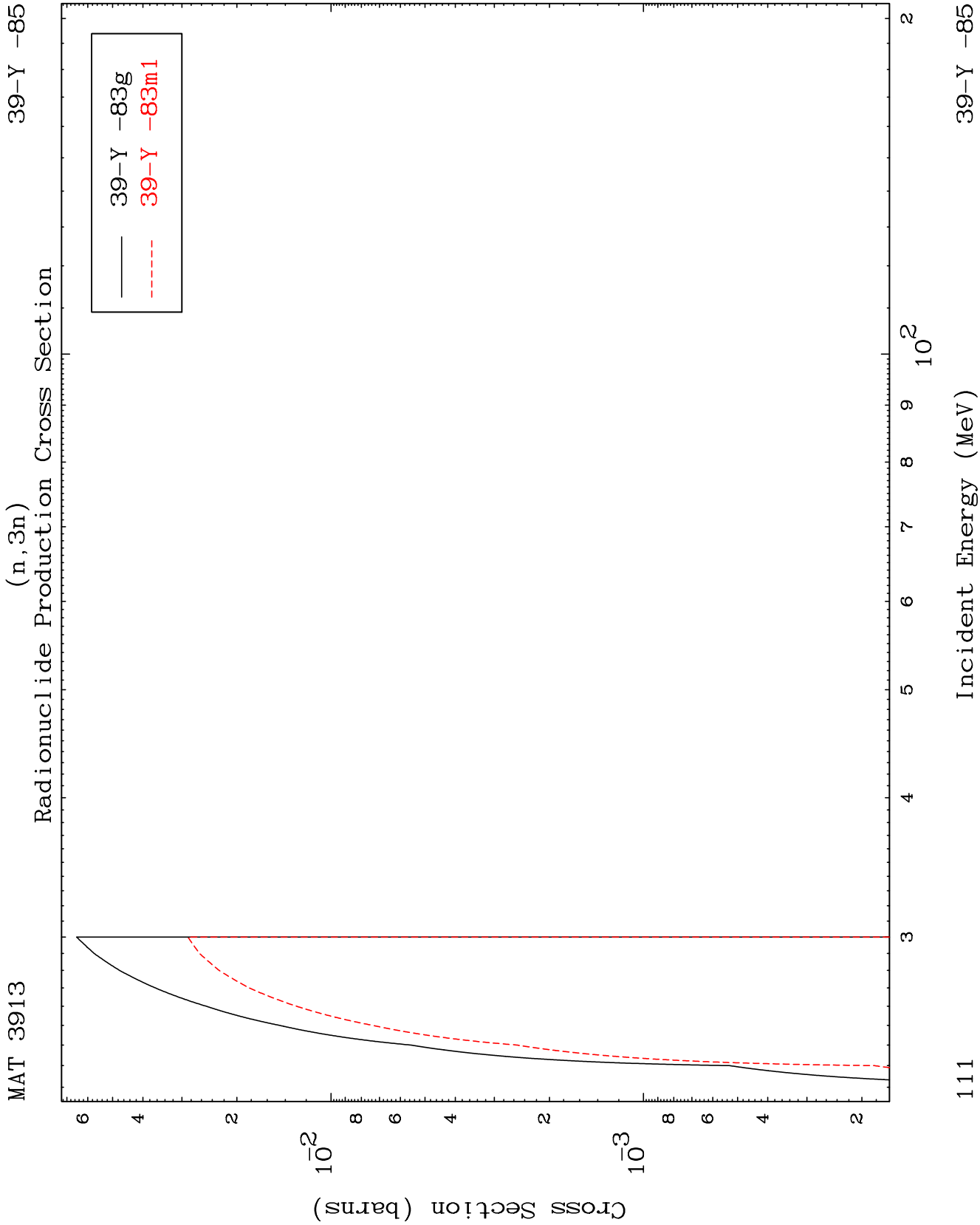
39-Y -85

Radionuclide Production Cross Section
(n,2n)



110

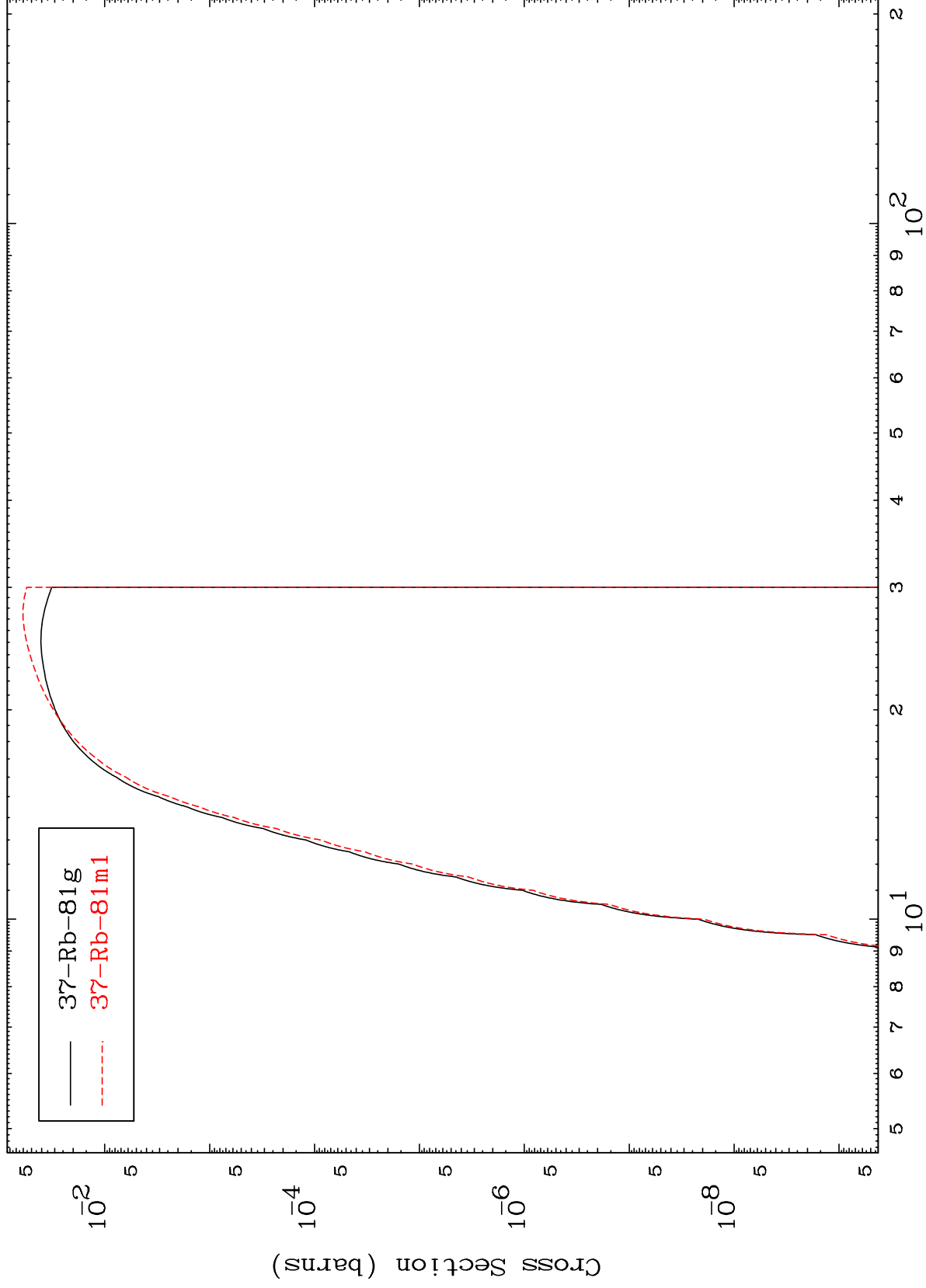
39-Y -85



MAT 3913

39-Y -85

$(n,n') \alpha$
Radionuclide Production Cross Section



39-Y -85

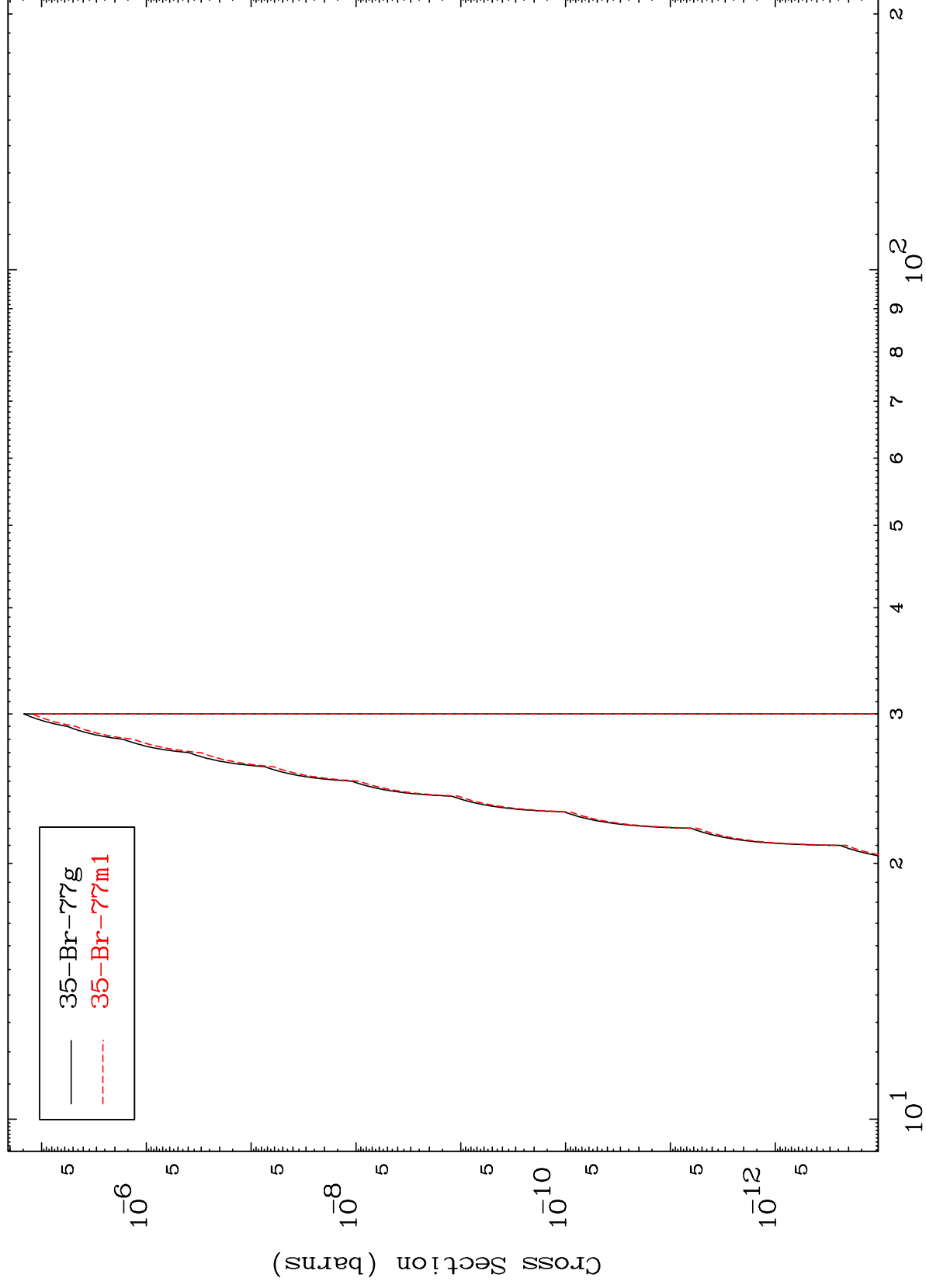
Incident Energy (MeV)

112

MAT 3913

39-Y -85

(n,n') 2 α
Radionuclide Production Cross Section

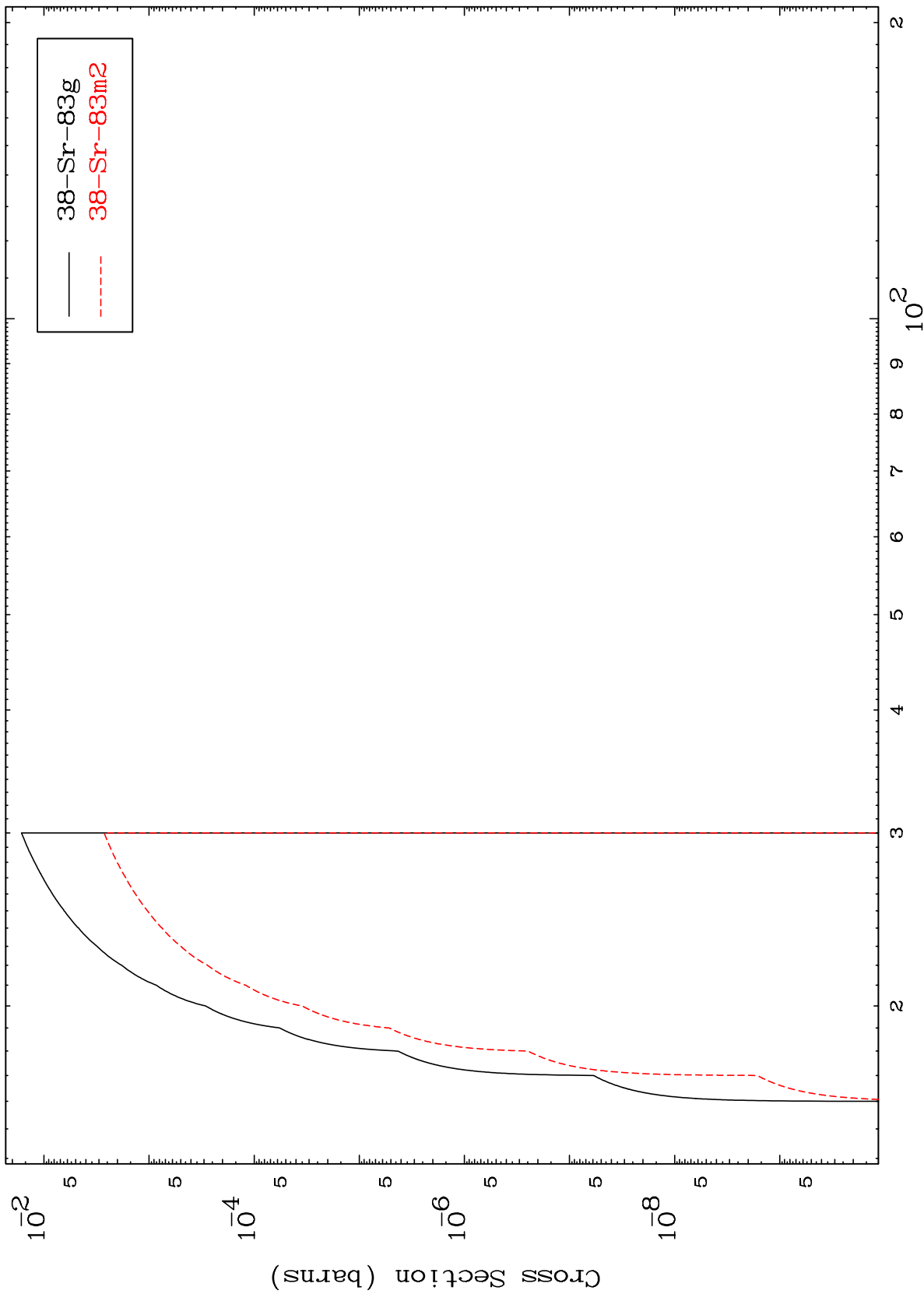


39-Y -85

Incident Energy (MeV)

113

Radionuclide Production Cross Section

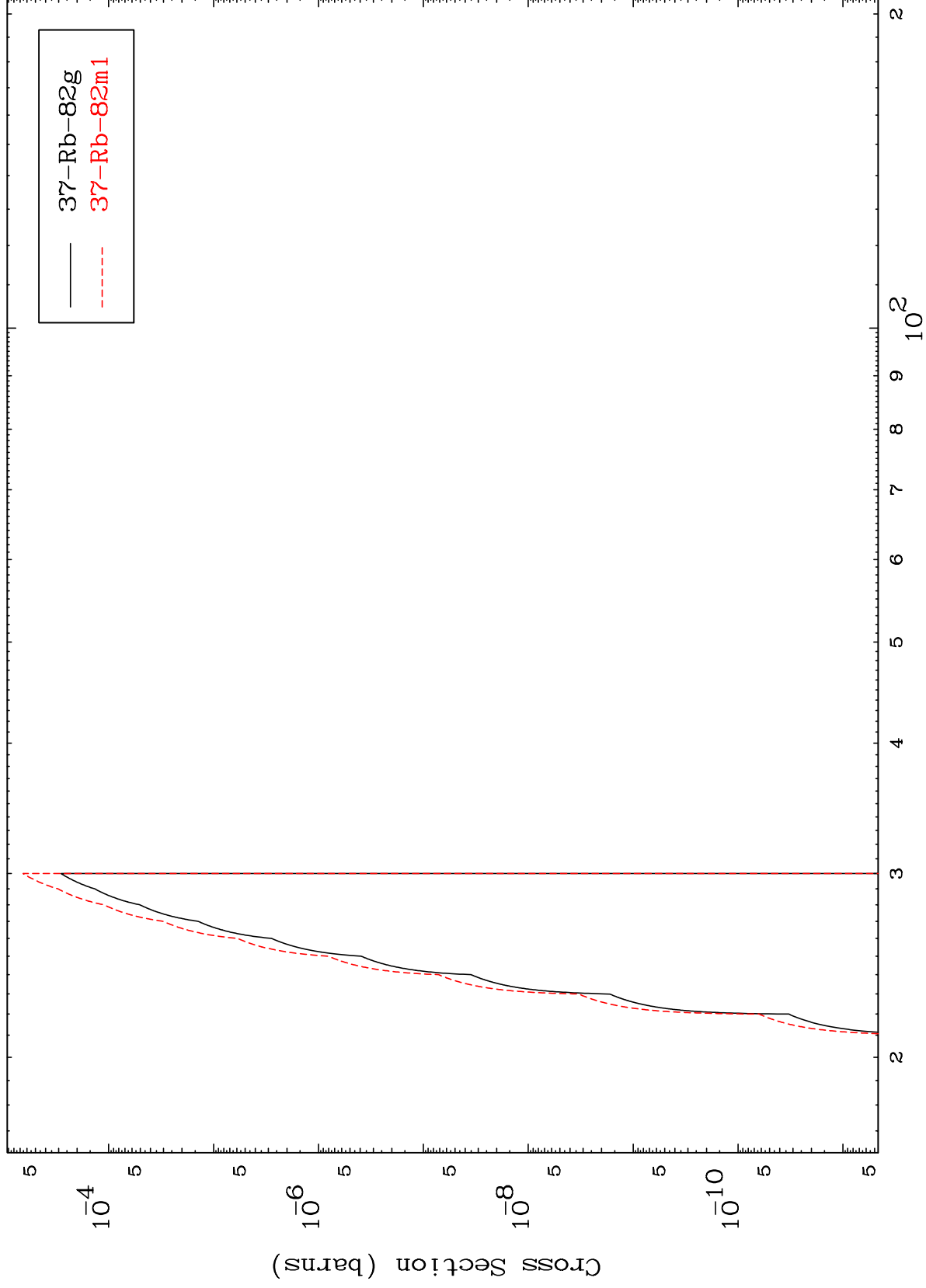


MAT 3913

(n,n') He-3

39-Y -85

Radionuclide Production Cross Section



115

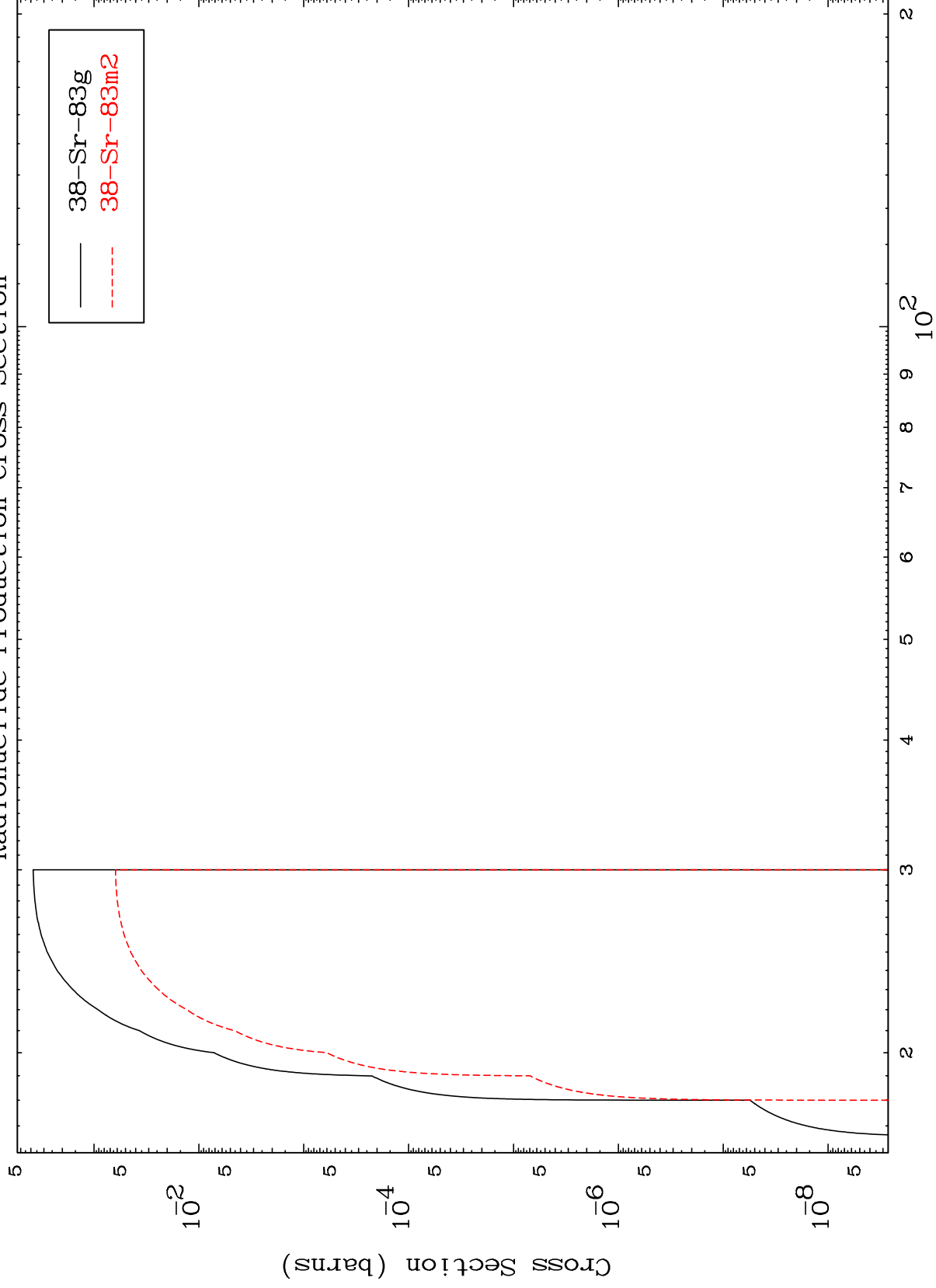
Incident Energy (MeV)

39-Y -85

MAT 3913

39-Y -85

(n,2n) p
Radionuclide Production Cross Section

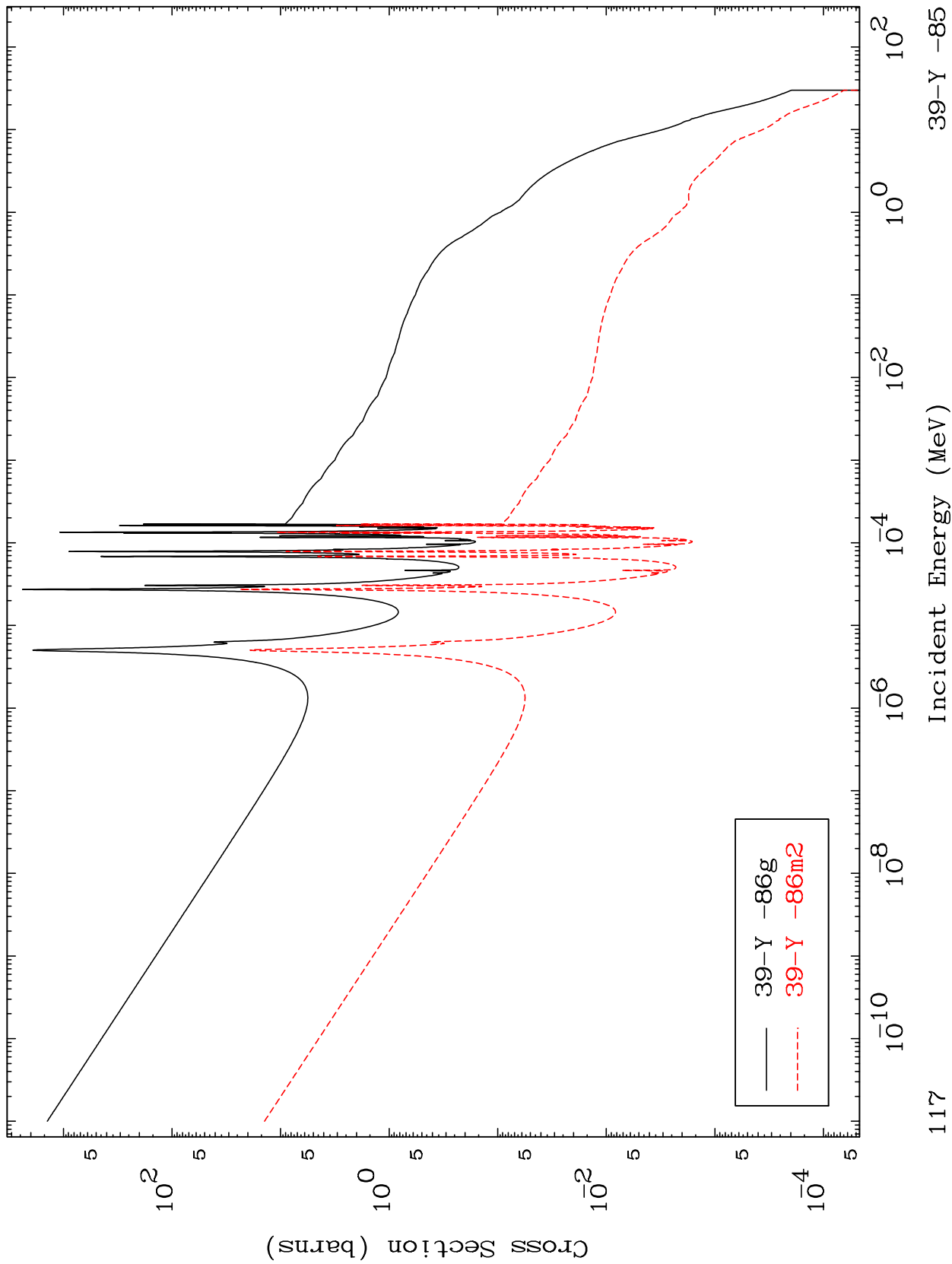


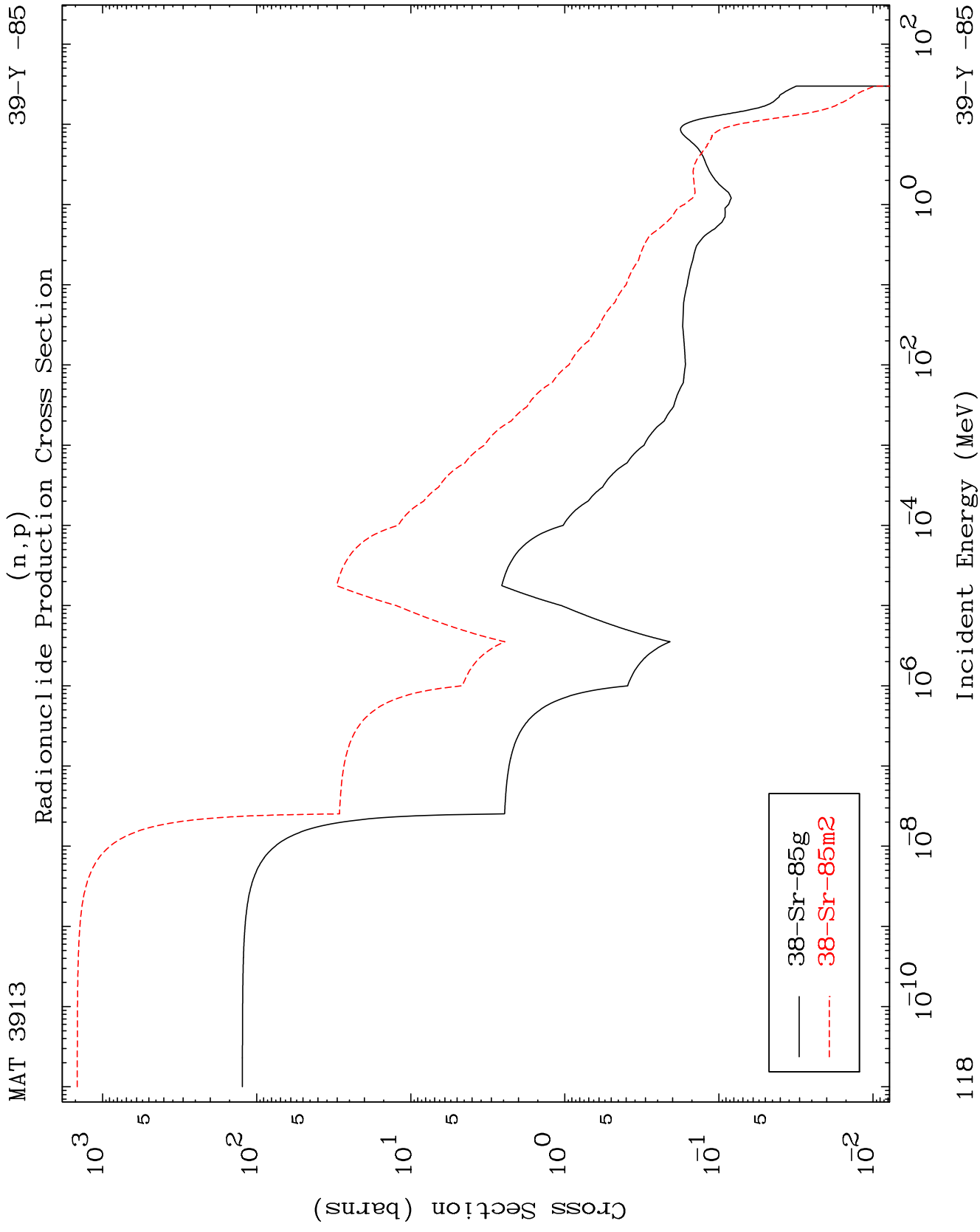
116

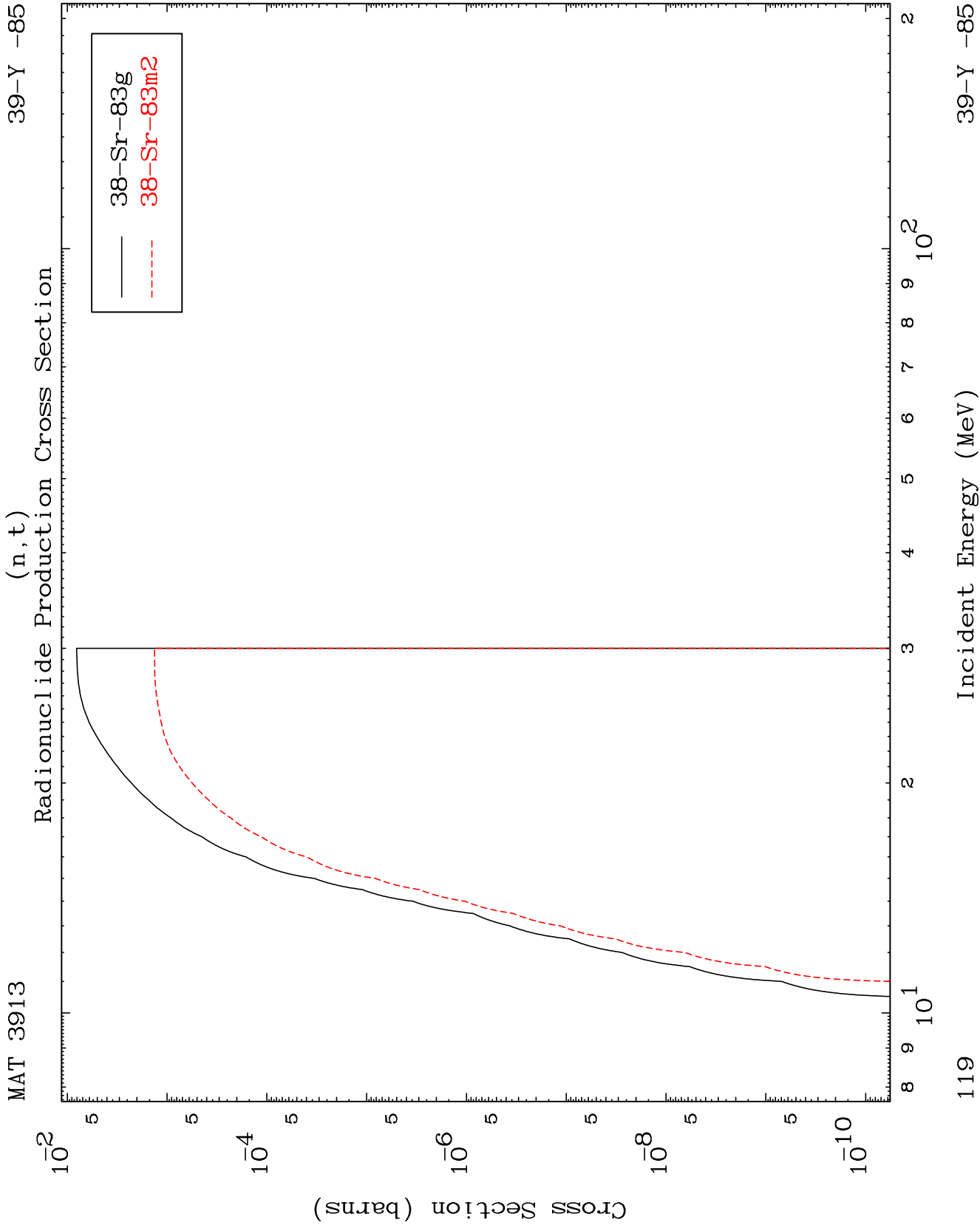
Incident Energy (MeV)

39-Y -85

Radionuclide Production Cross Section
(n, γ)





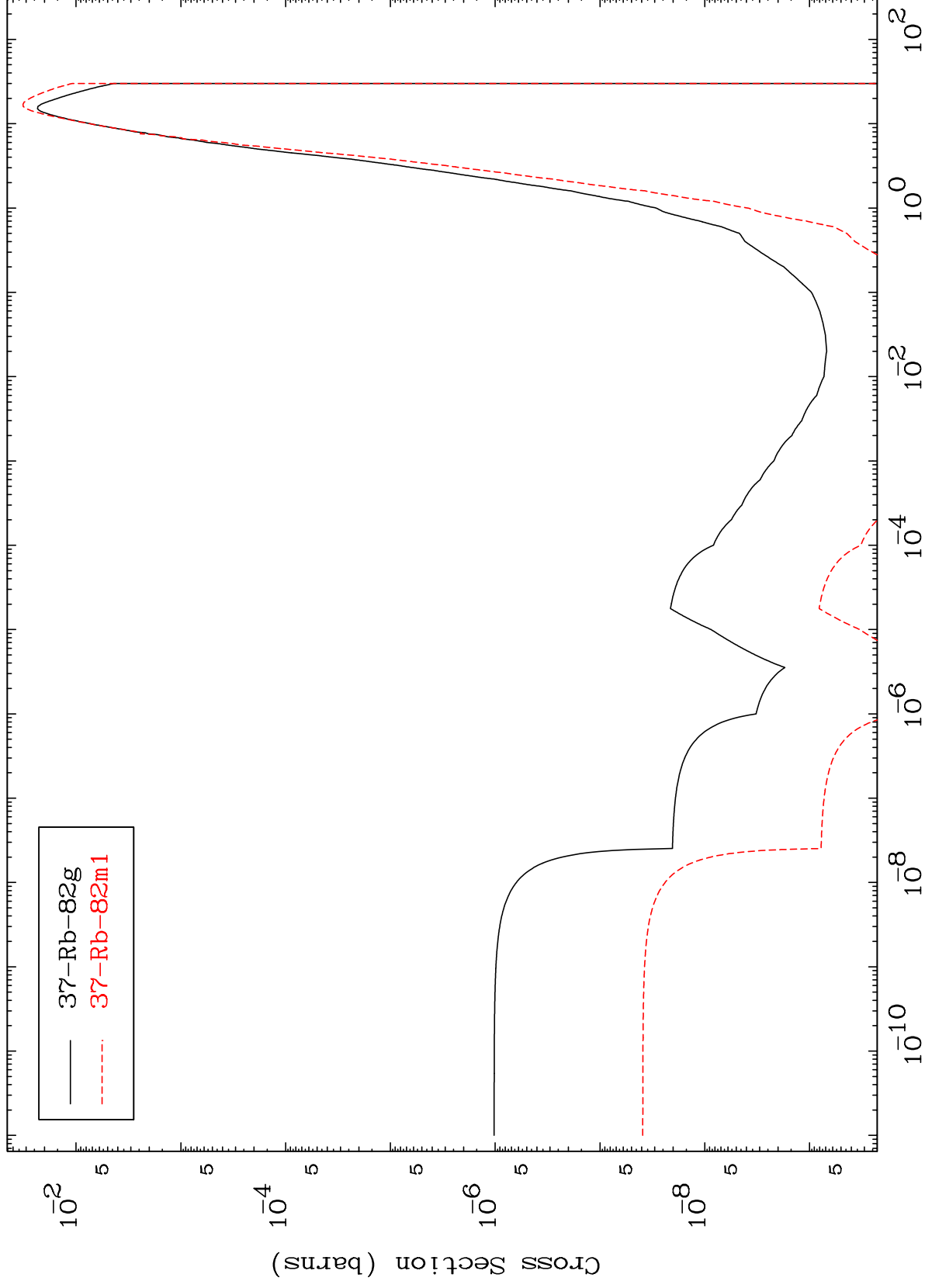


MAT 3913

(n, α)

39-Y -85

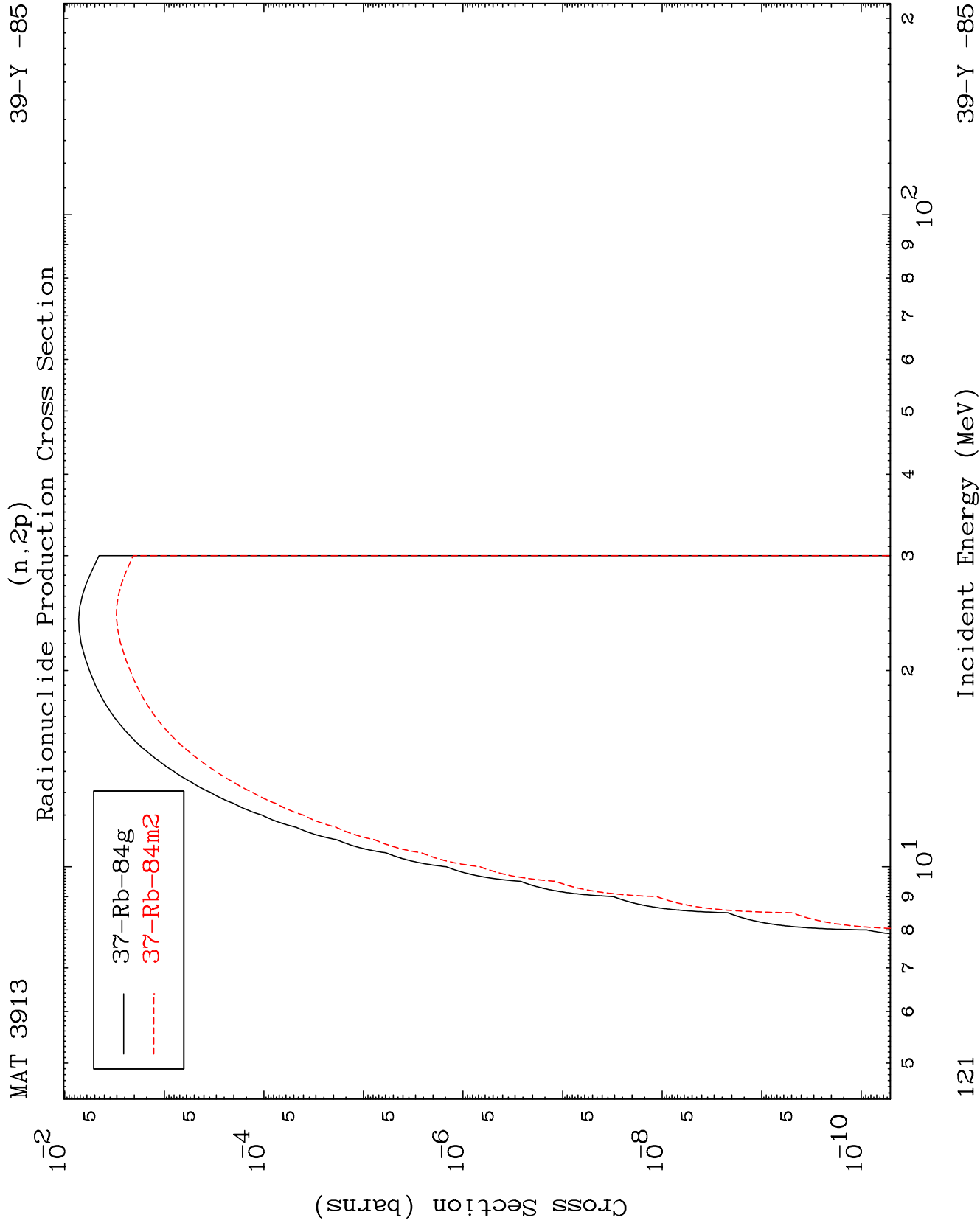
Radionuclide Production Cross Section



120

Incident Energy (MeV)

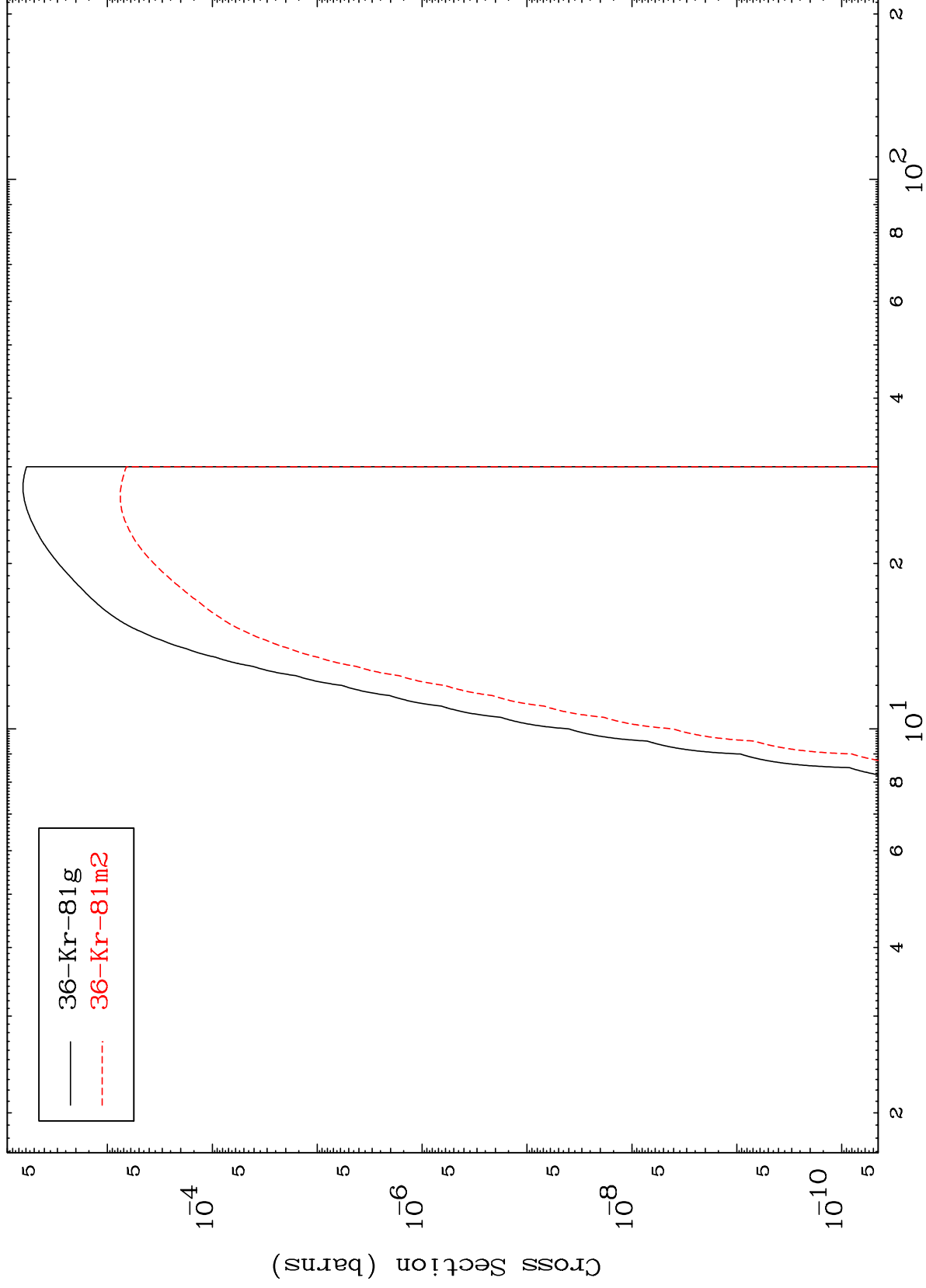
39-Y -85



MAT 3913

39-Y -85

(n,p) α
Radionuclide Production Cross Section



122

Incident Energy (MeV)

39-Y -85

Radionuclide Production Cross Section

