

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

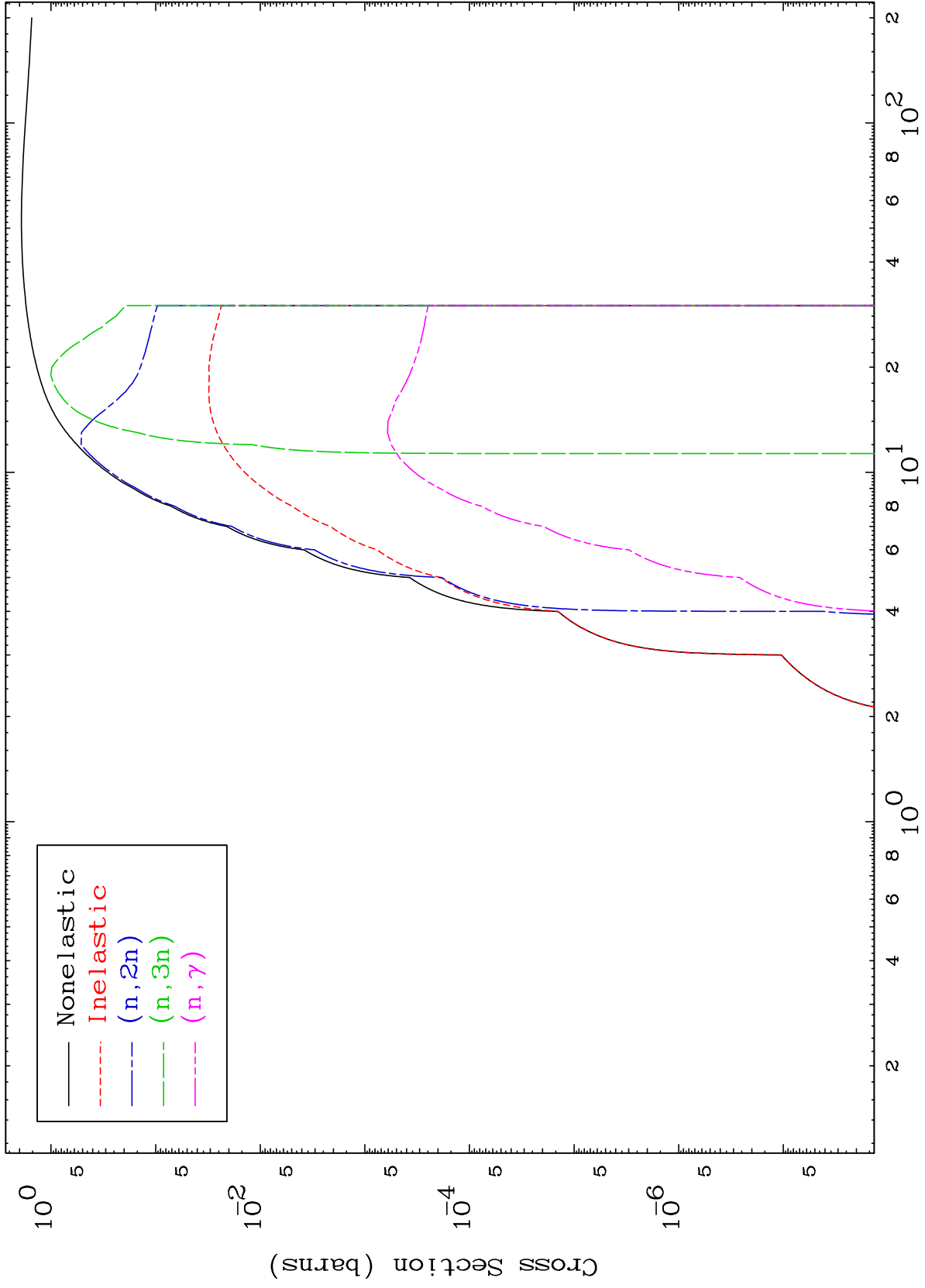
Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

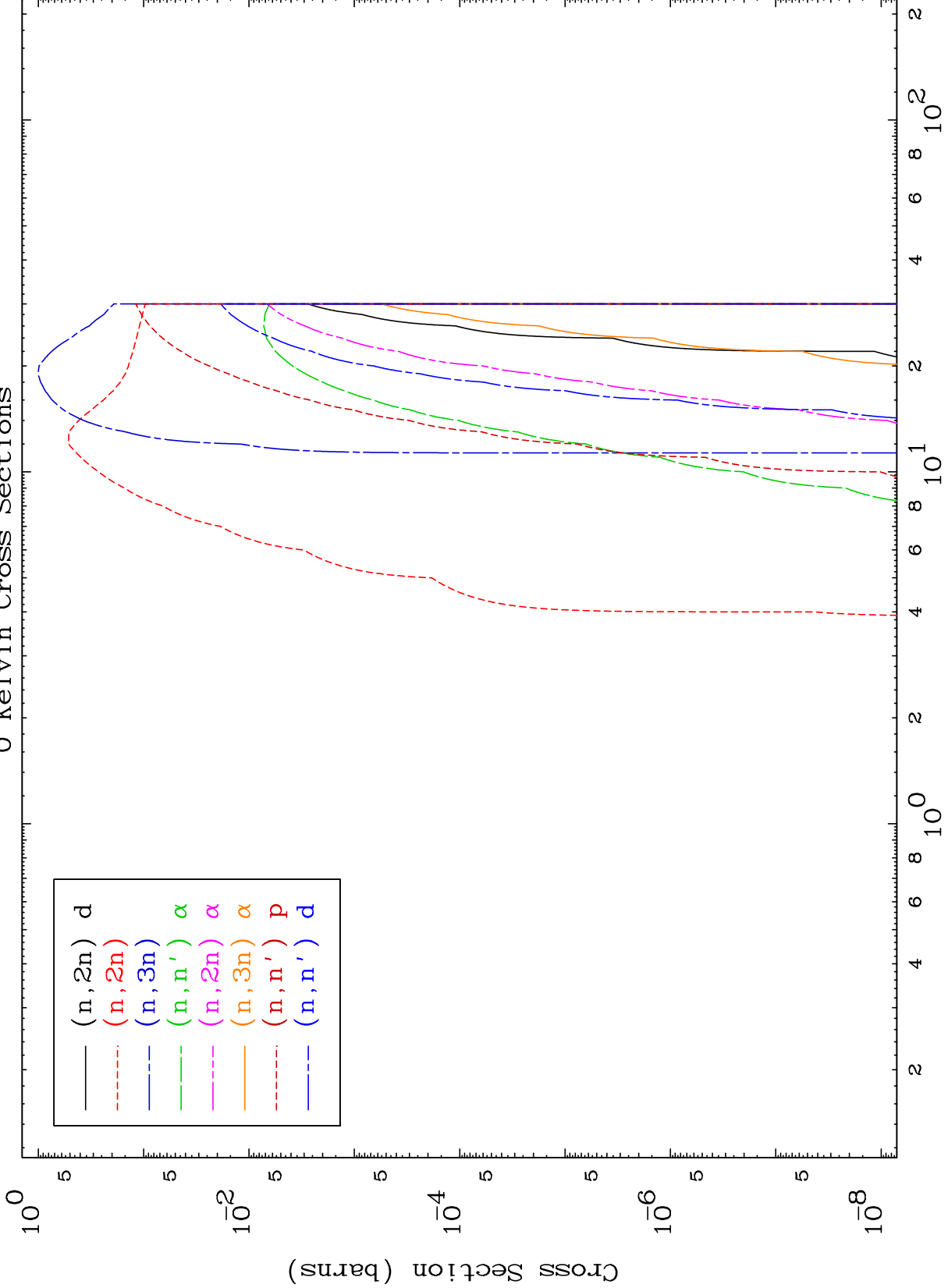
0 Kelvin Cross Sections



MAT 7143

Proton Neutron Absorption  
0 Kelvin Cross Sections

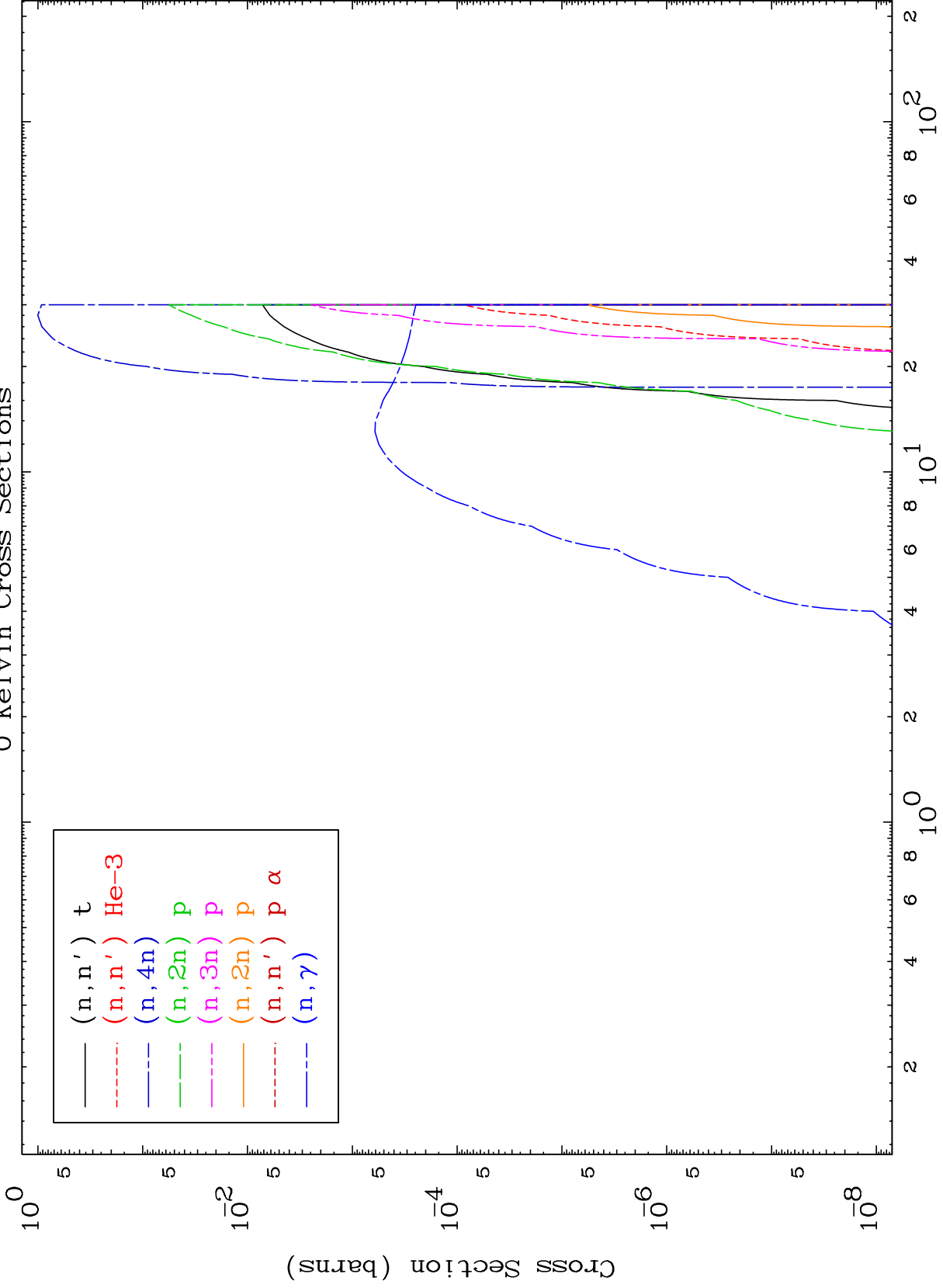
71-Lu-181

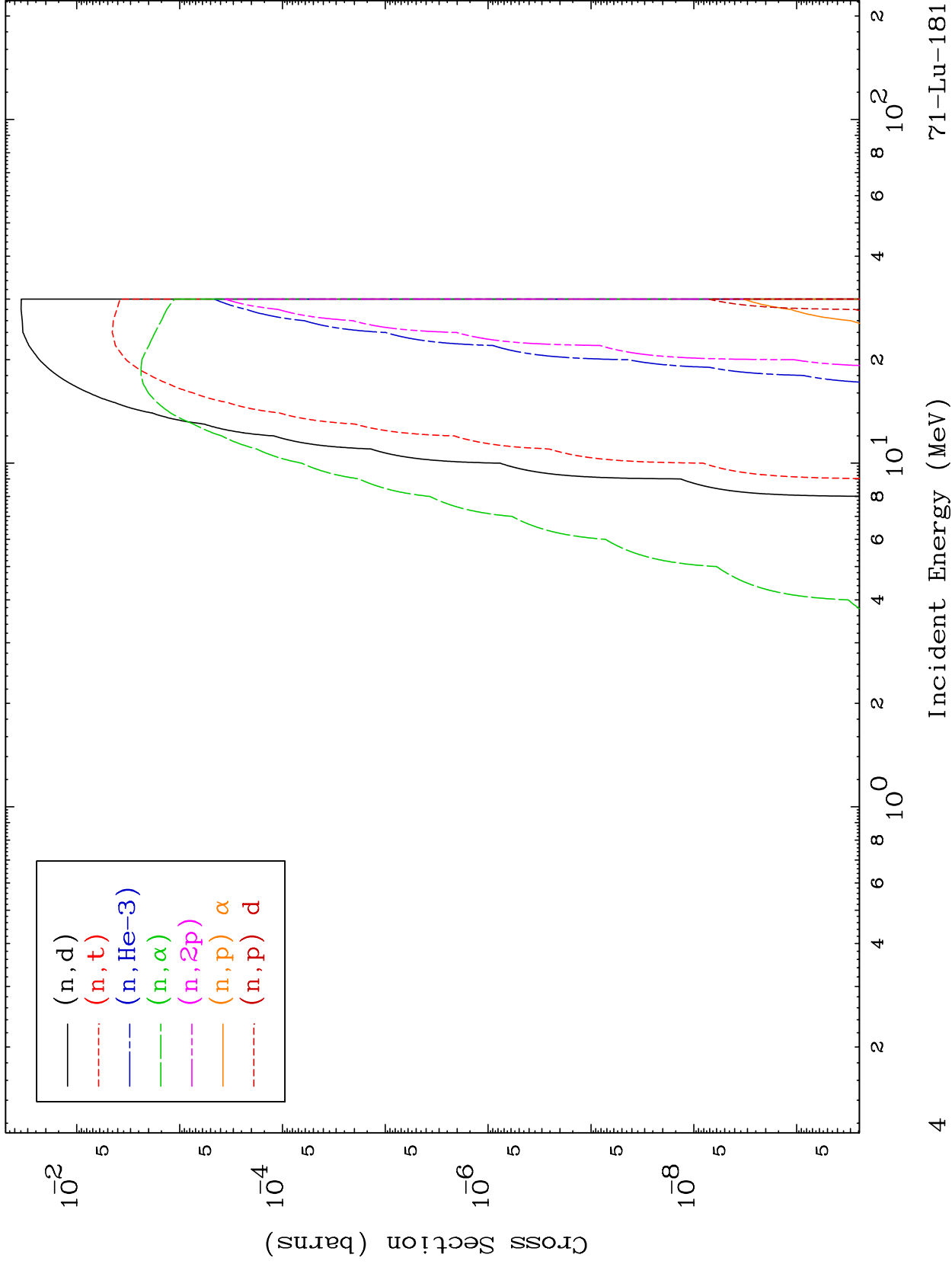


MAT 7143

Proton Neutron Absorption  
0 Kelvin Cross Sections

71-Lu-181

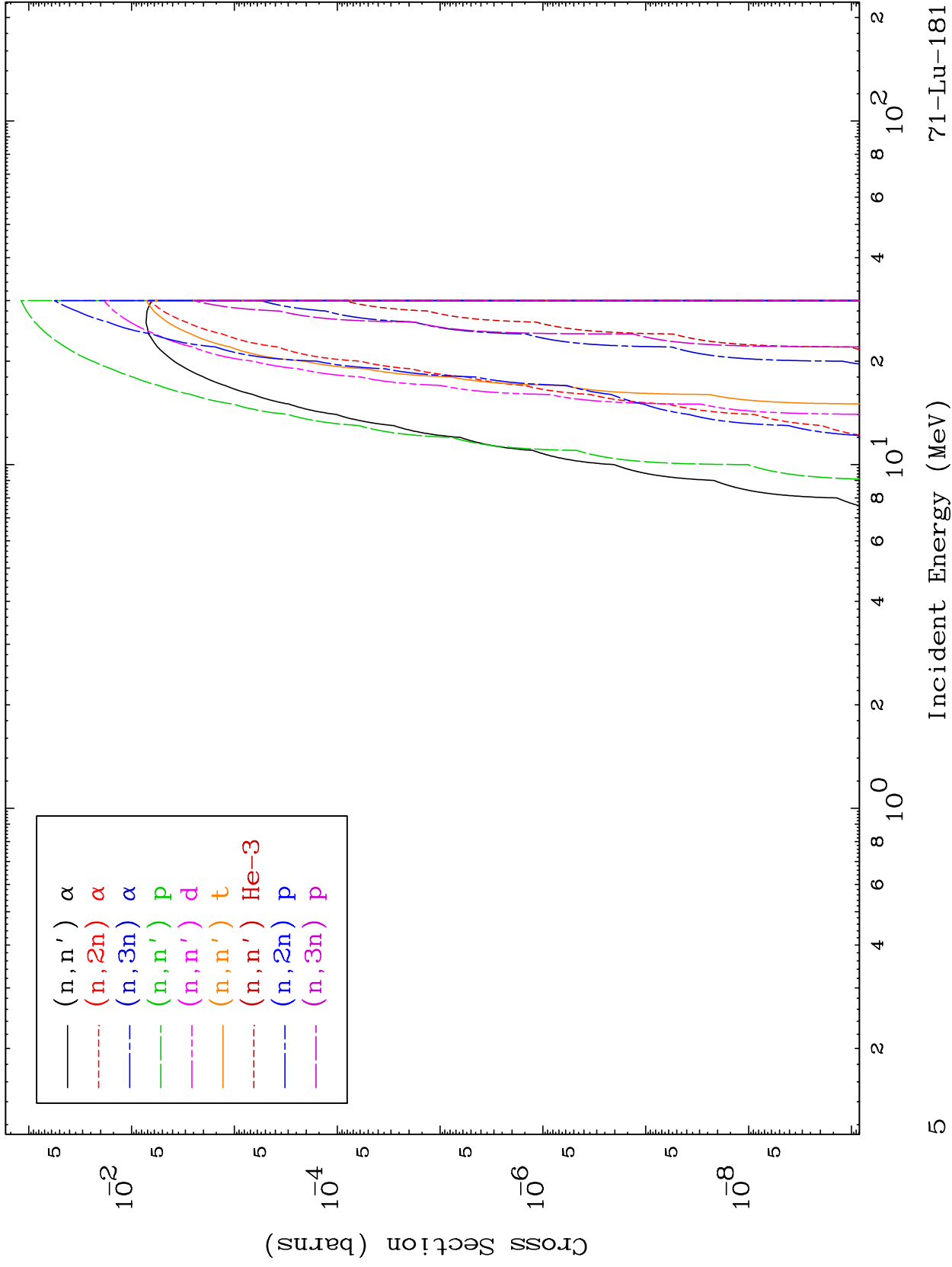


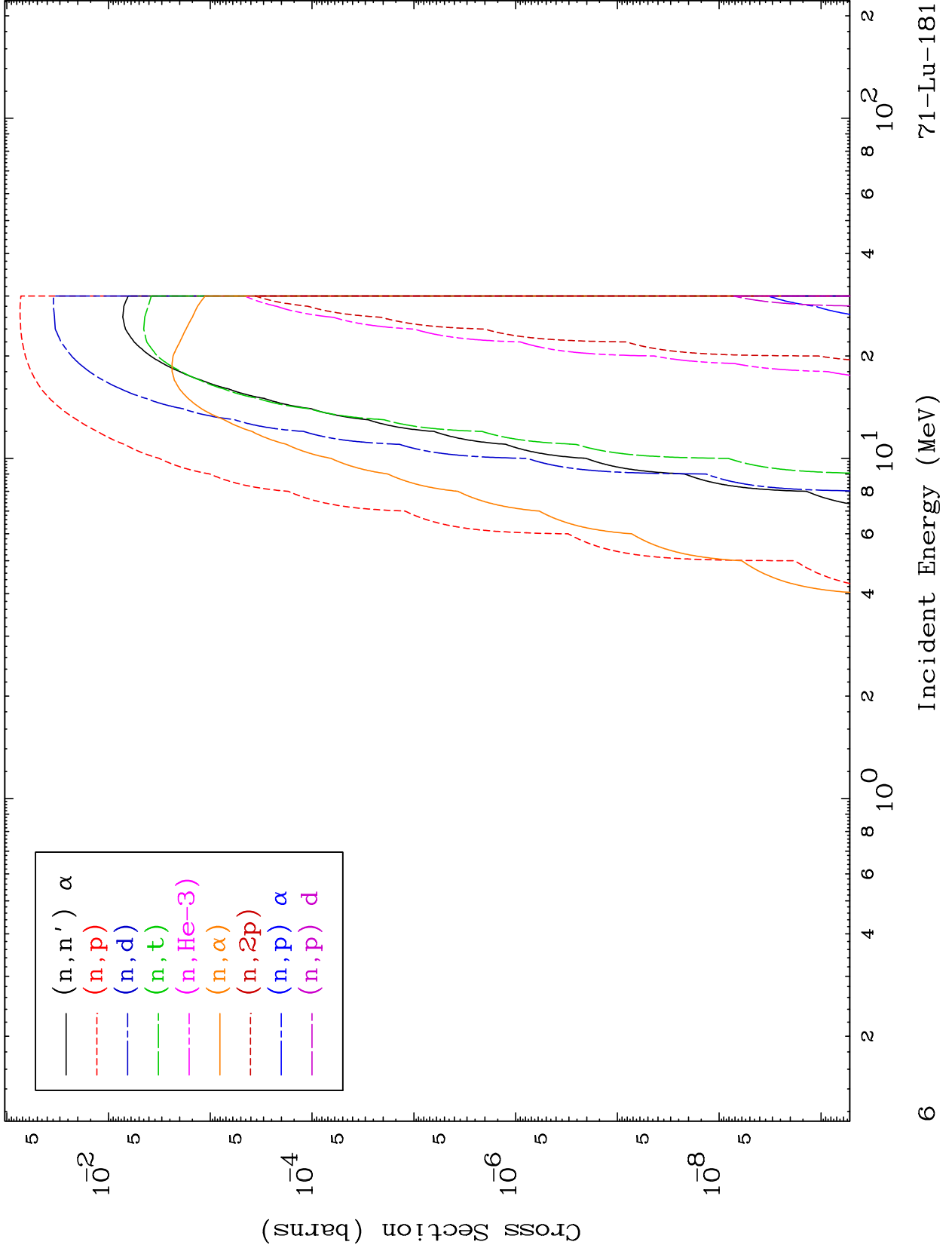


MAT 7143

Proton Charged Particle  
0 Kelvin Cross Sections

71-Lu-181

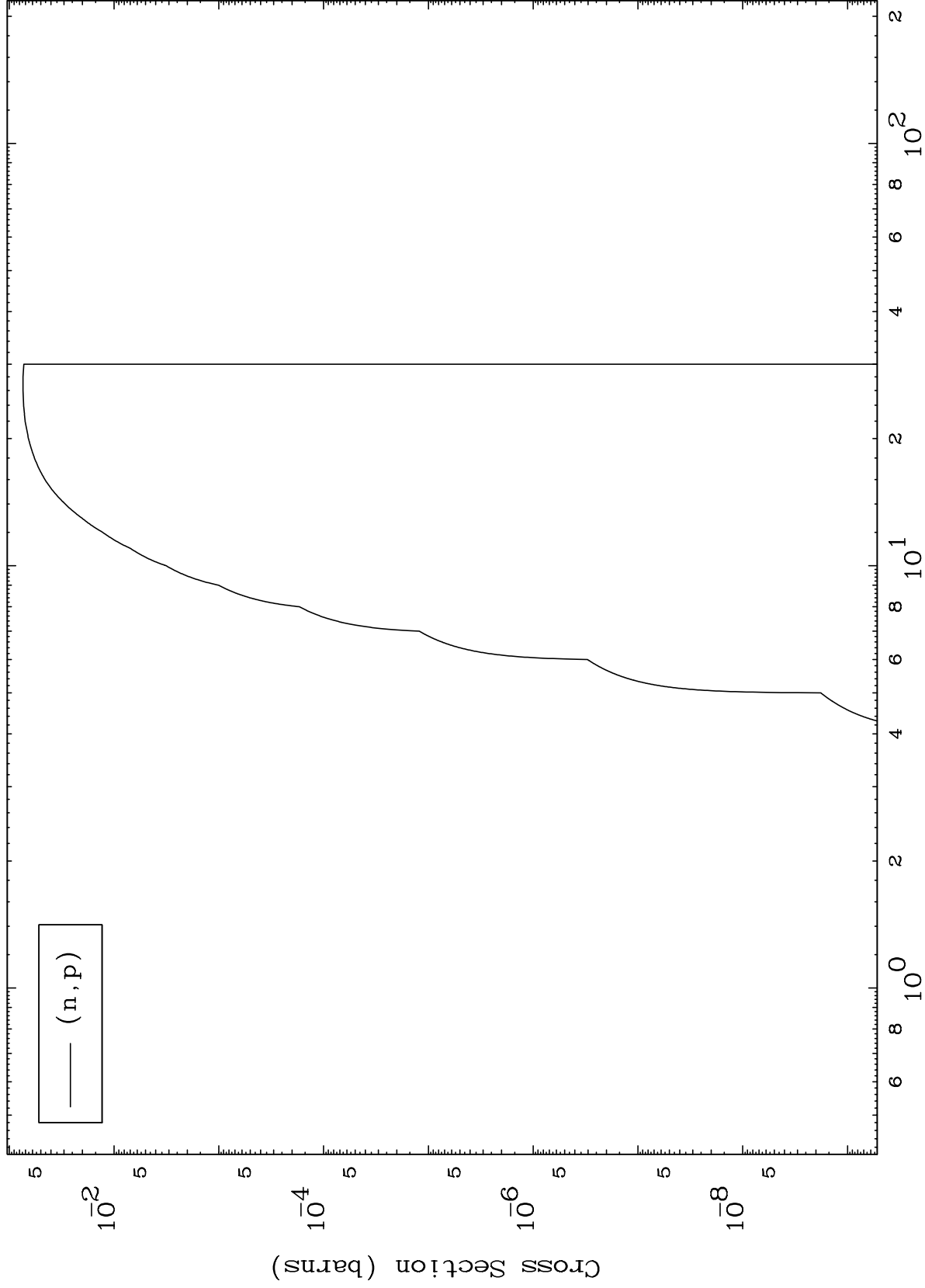




MAT 7143

(p,p) Levels  
0 Kelvin Cross Sections

71-Lu-181



7

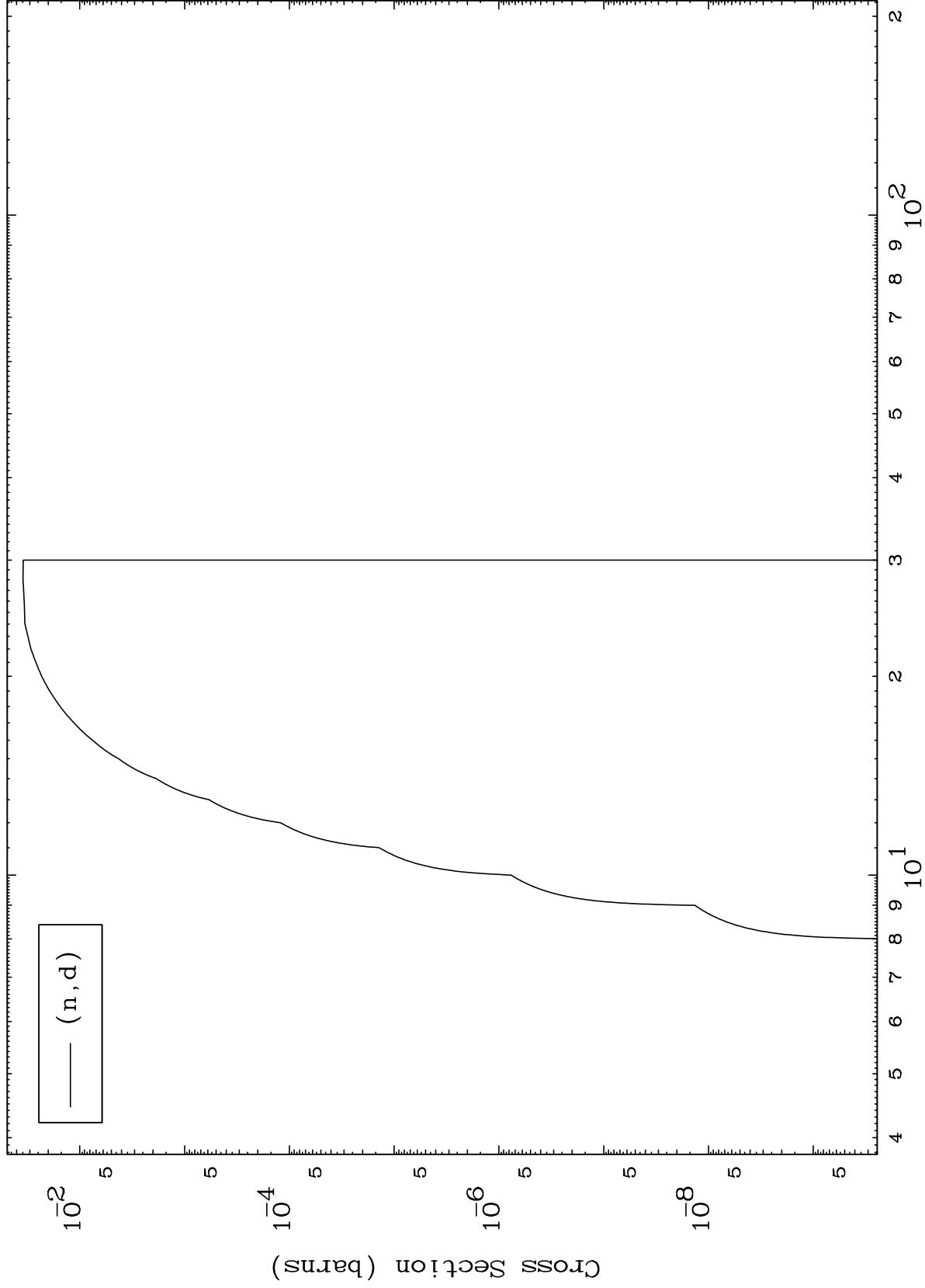
Incident Energy (MeV)

71-Lu-181

MAT 7143

71-Lu-181

(p,d) Levels  
0 Kelvin Cross Sections



8

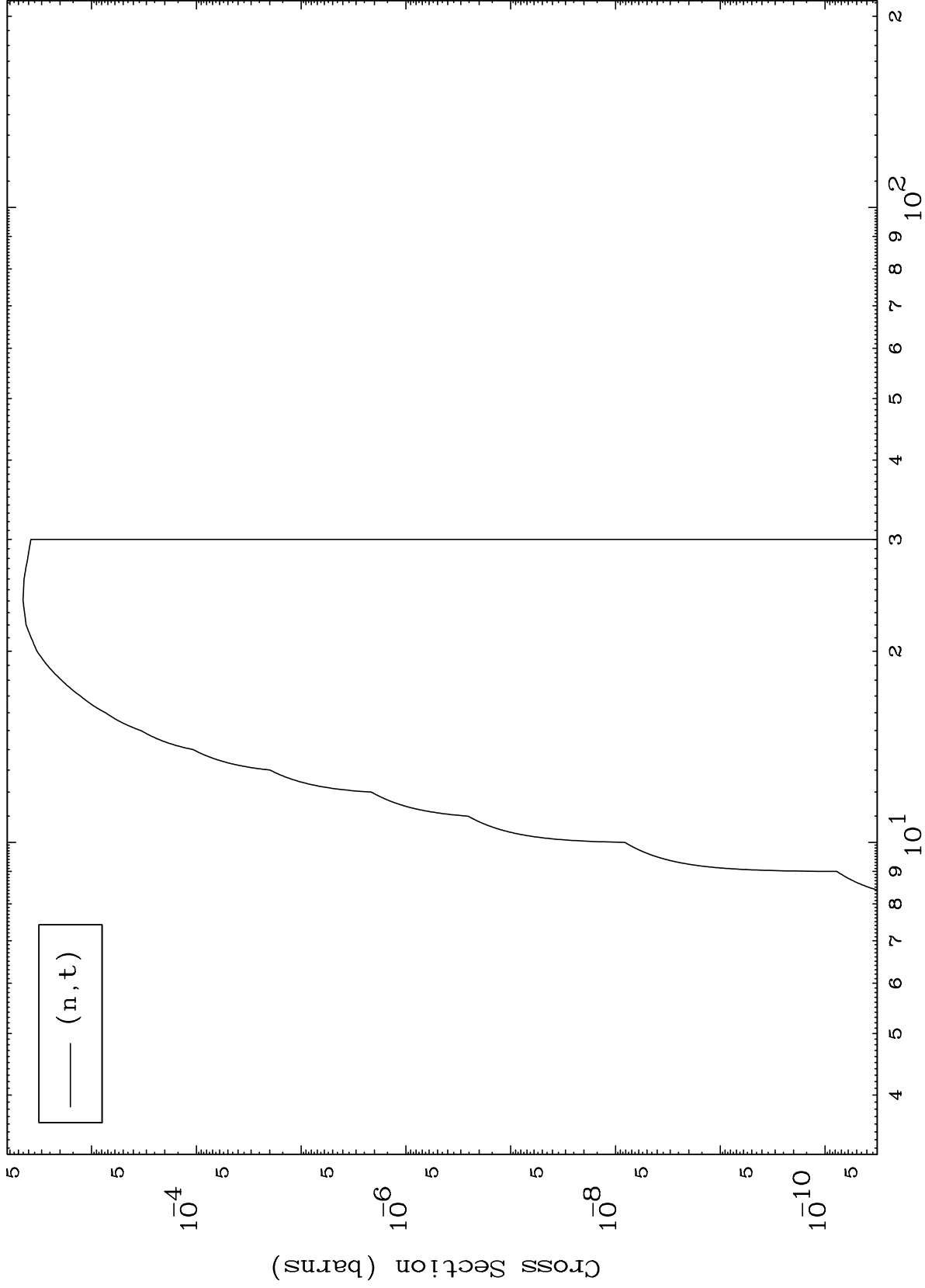
Incident Energy (MeV)

71-Lu-181

MAT 7143

71-Lu-181

(p,t) Levels  
0 Kelvin Cross Sections



9

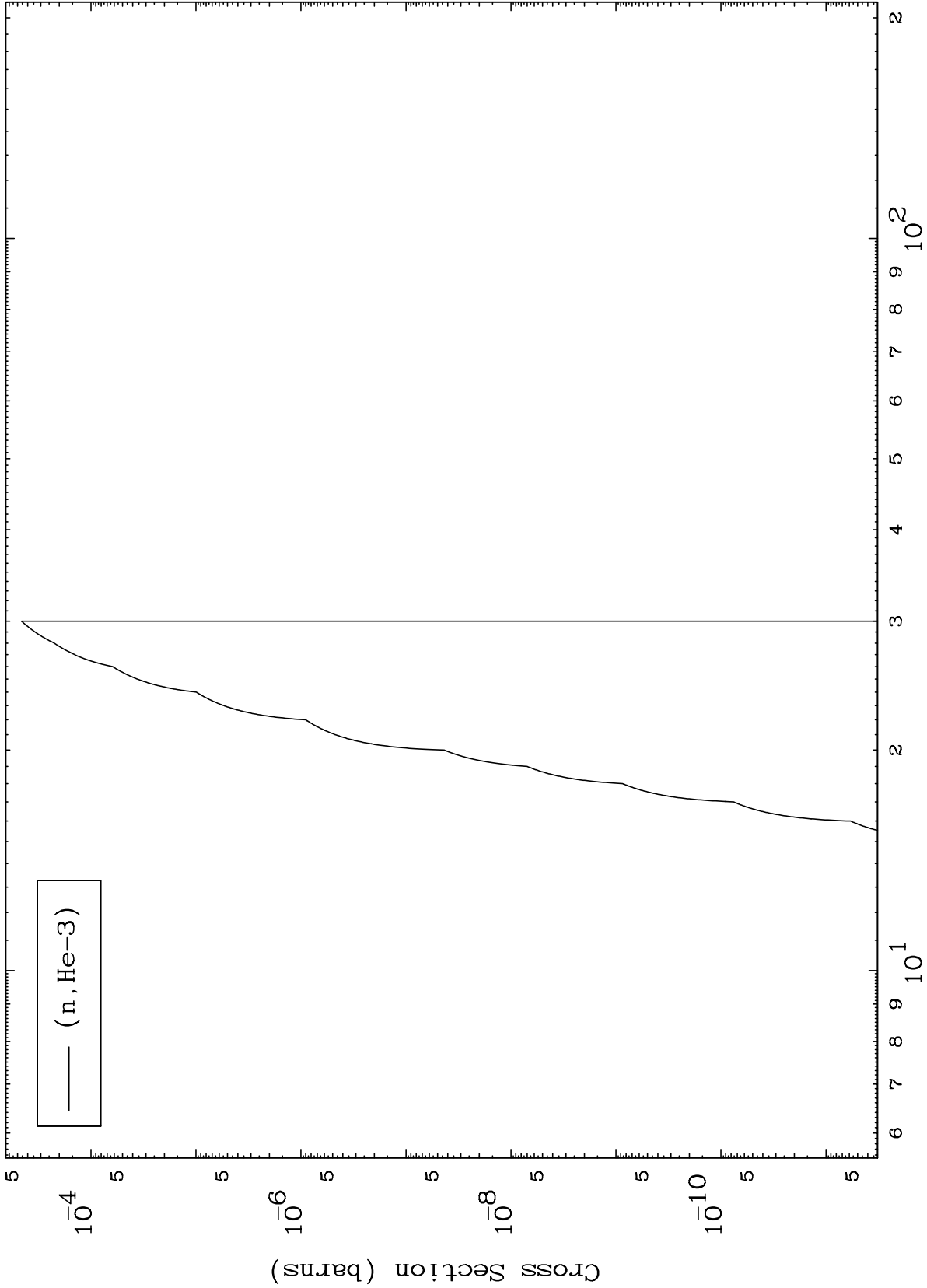
Incident Energy (MeV)

71-Lu-181

MAT 7143

71-Lu-181

(p,He3) Levels  
0 Kelvin Cross Sections

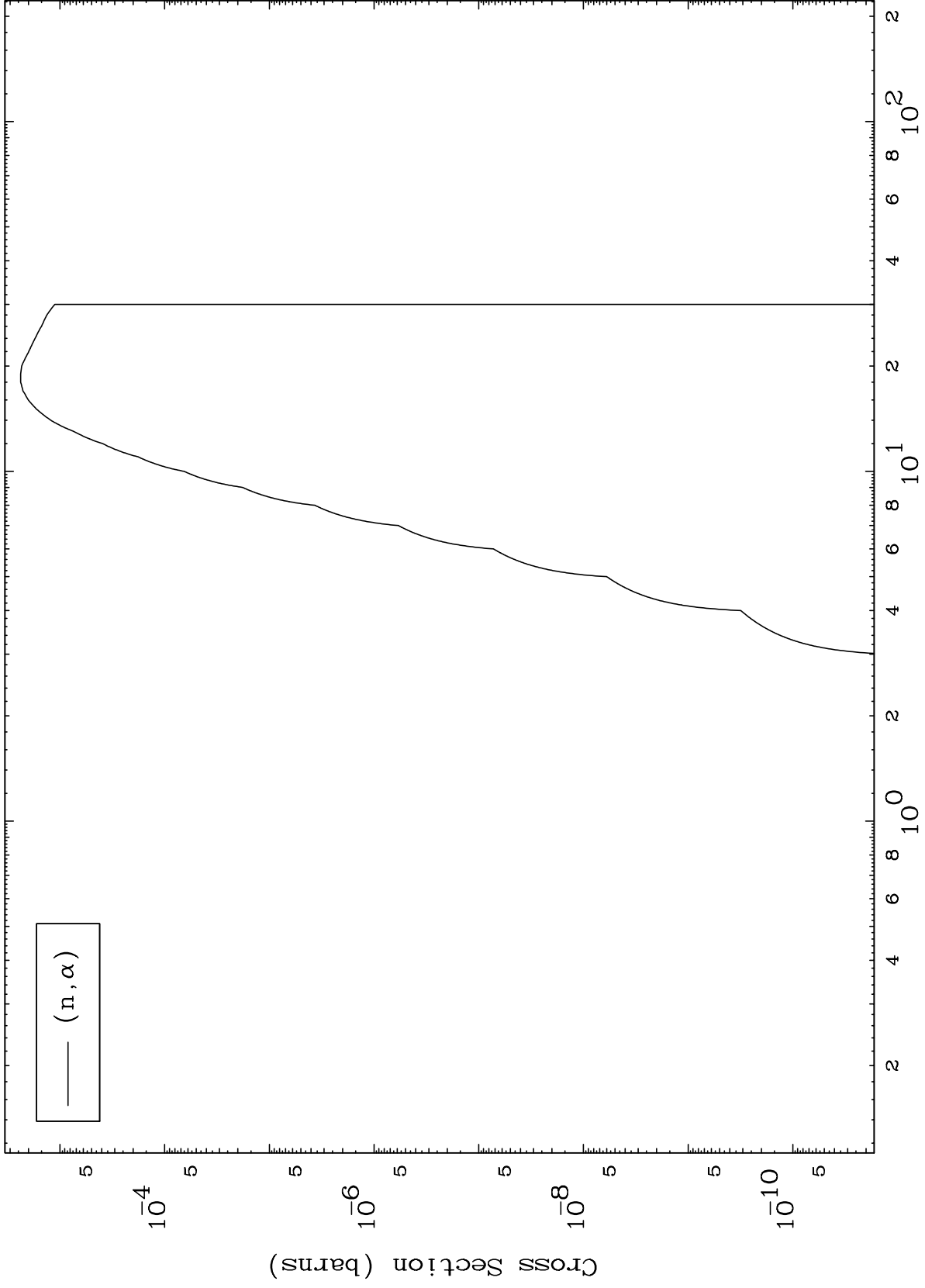


10

Incident Energy (MeV)

71-Lu-181

(p,  $\alpha$ ) Levels  
0 Kelvin Cross Sections

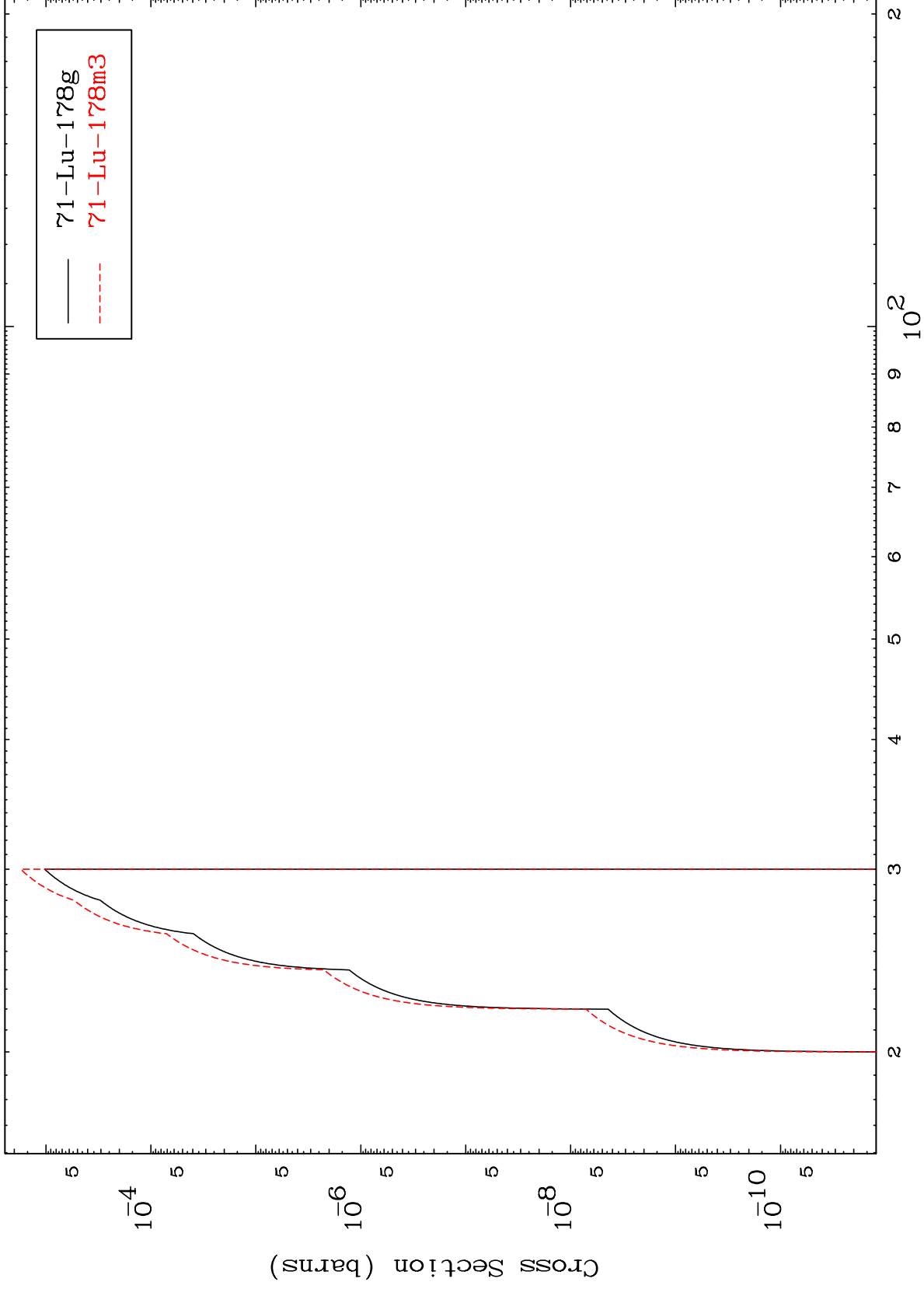


MAT 7143

(n,2n) d

71-Lu-181

Radionuclide Production Cross Section



12

Incident Energy (MeV)

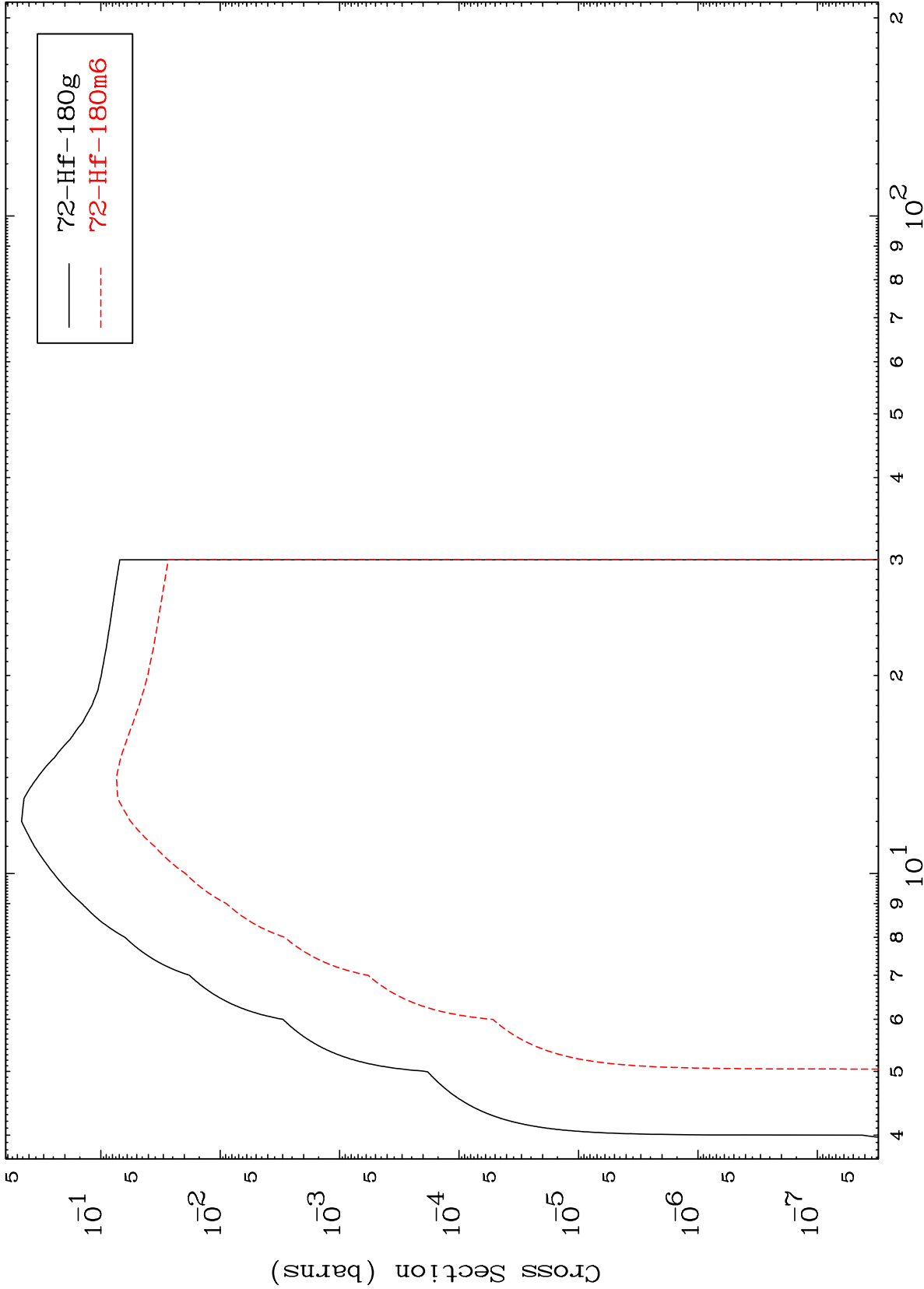
71-Lu-181

MAT 7143

(n,2n)

71-Lu-181

Radionuclide Production Cross Section



13

Incident Energy (MeV)

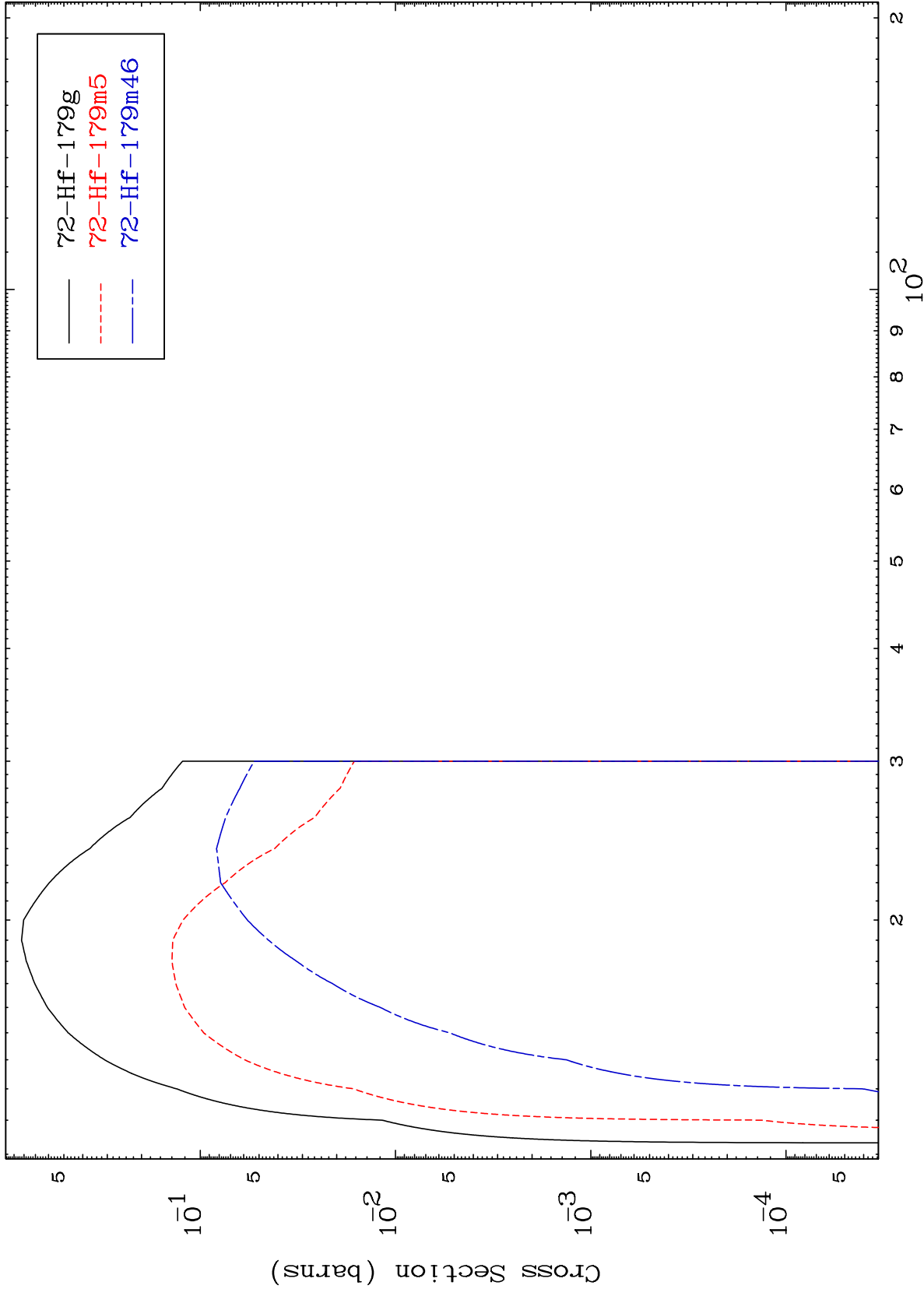
71-Lu-181

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(n,3n)

71-Lu-181

Radionuclide Production Cross Section



14

Incident Energy (MeV)

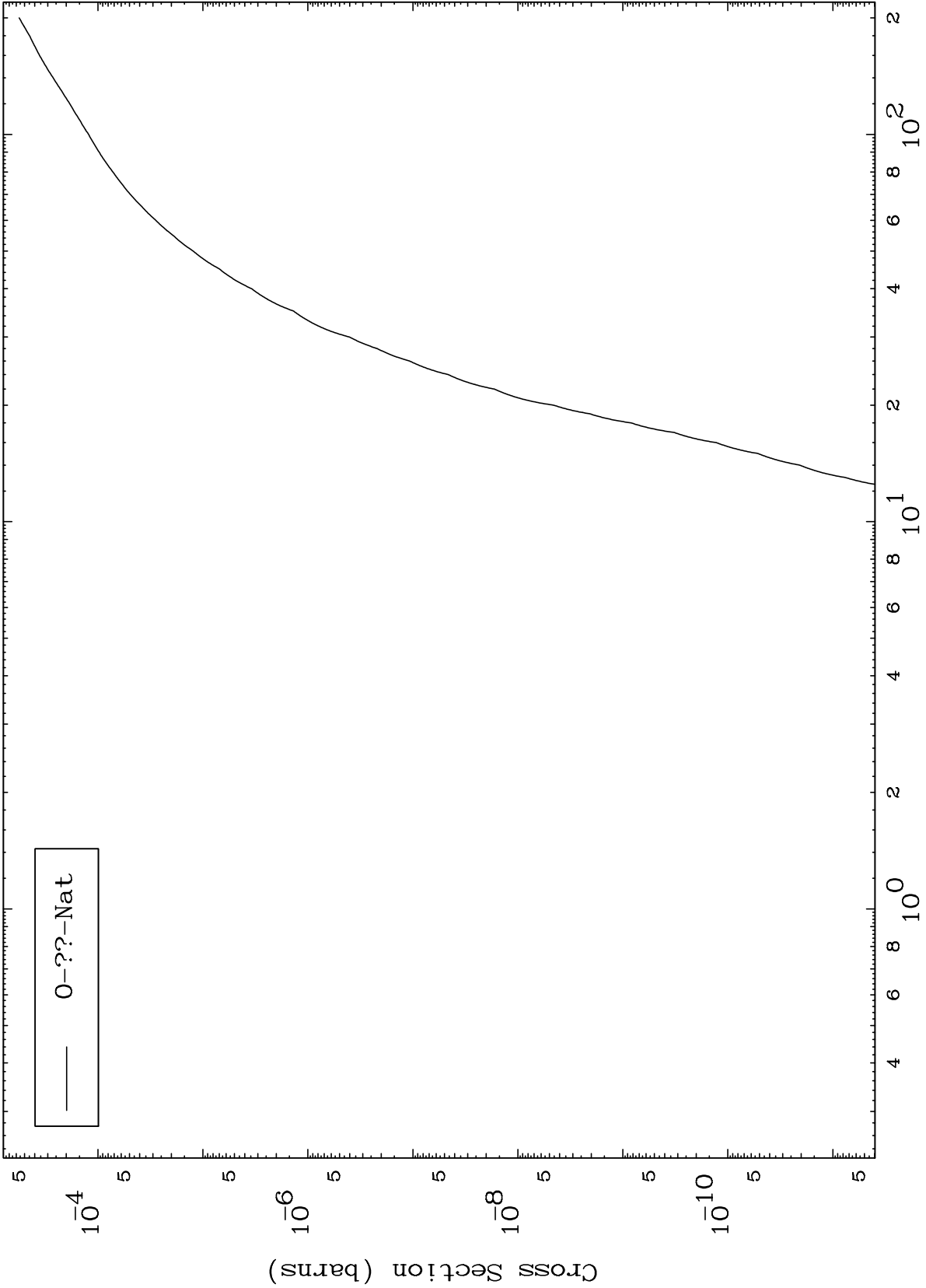
71-Lu-181

MAT 7143

Fission

<sup>71</sup>Lu-181

Radionuclide Production Cross Section



15

Incident Energy (MeV)

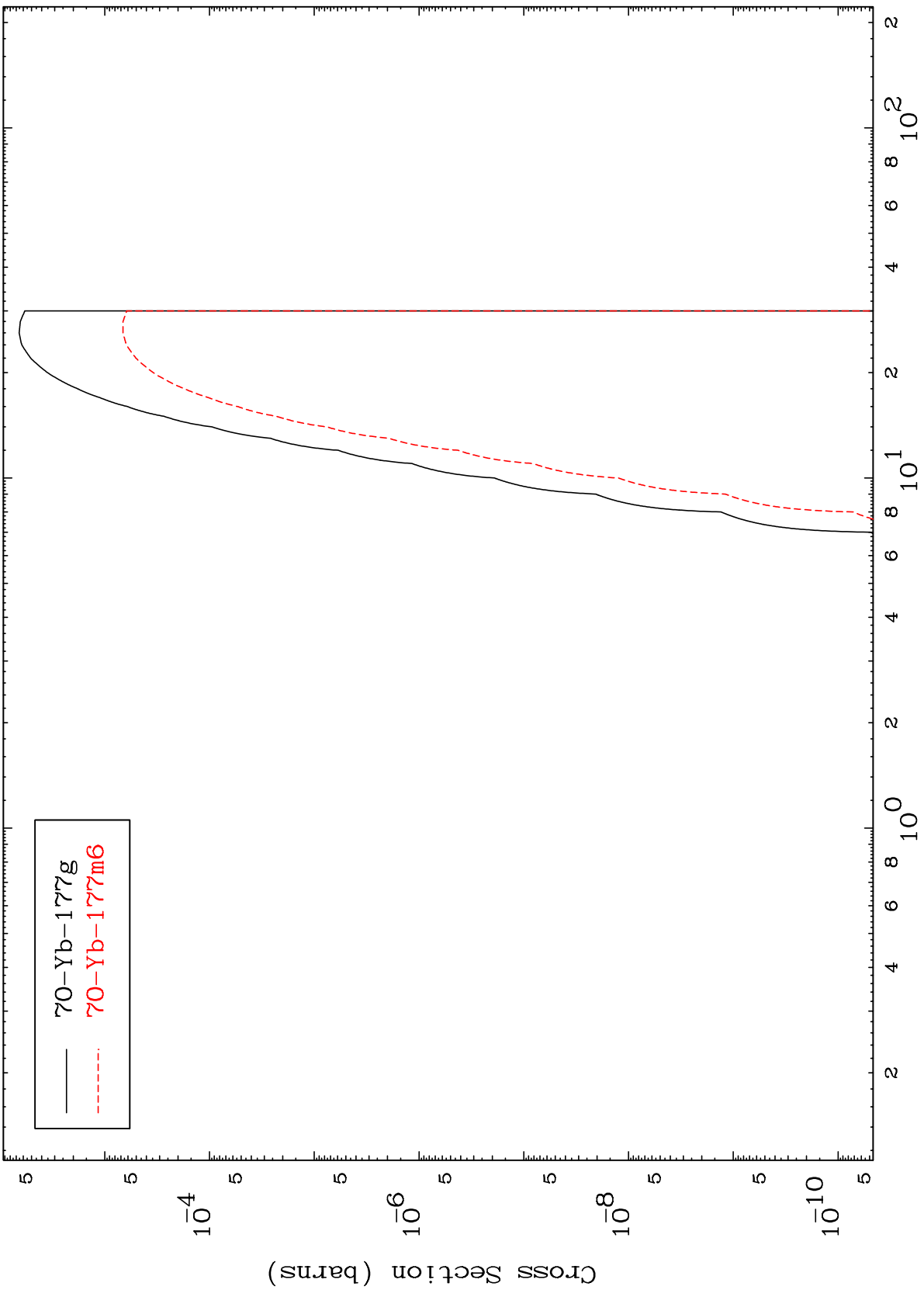
<sup>71</sup>Lu-181

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$(n, n') \alpha$

$^{71}\text{Lu-181}$

Radionuclide Production Cross Section



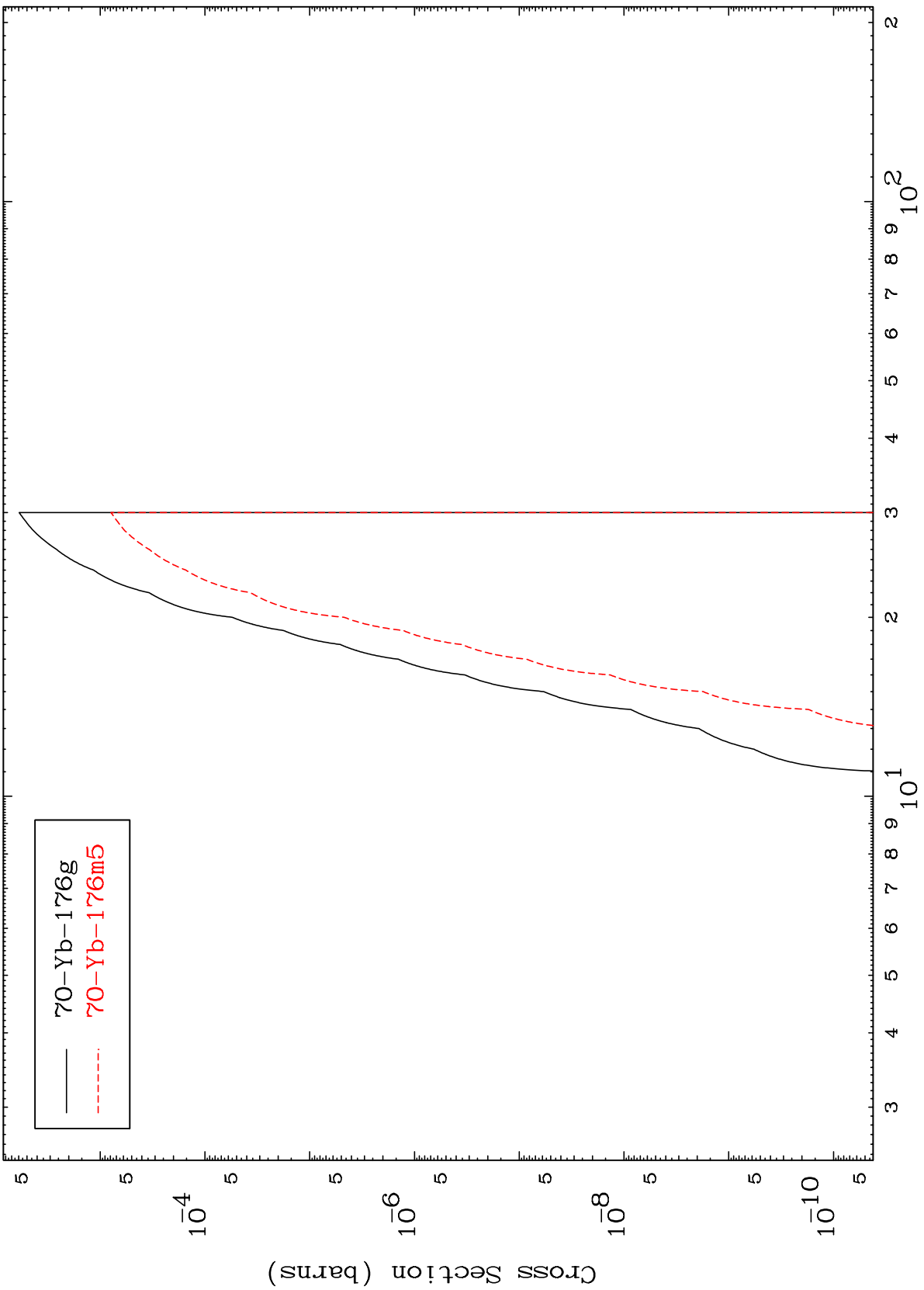
—  $^{70}\text{Yb-177g}$   
- - -  $^{70}\text{Yb-177m6}$

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$(n,2n) \alpha$

$^{71}\text{Lu-181}$

Radionuclide Production Cross Section



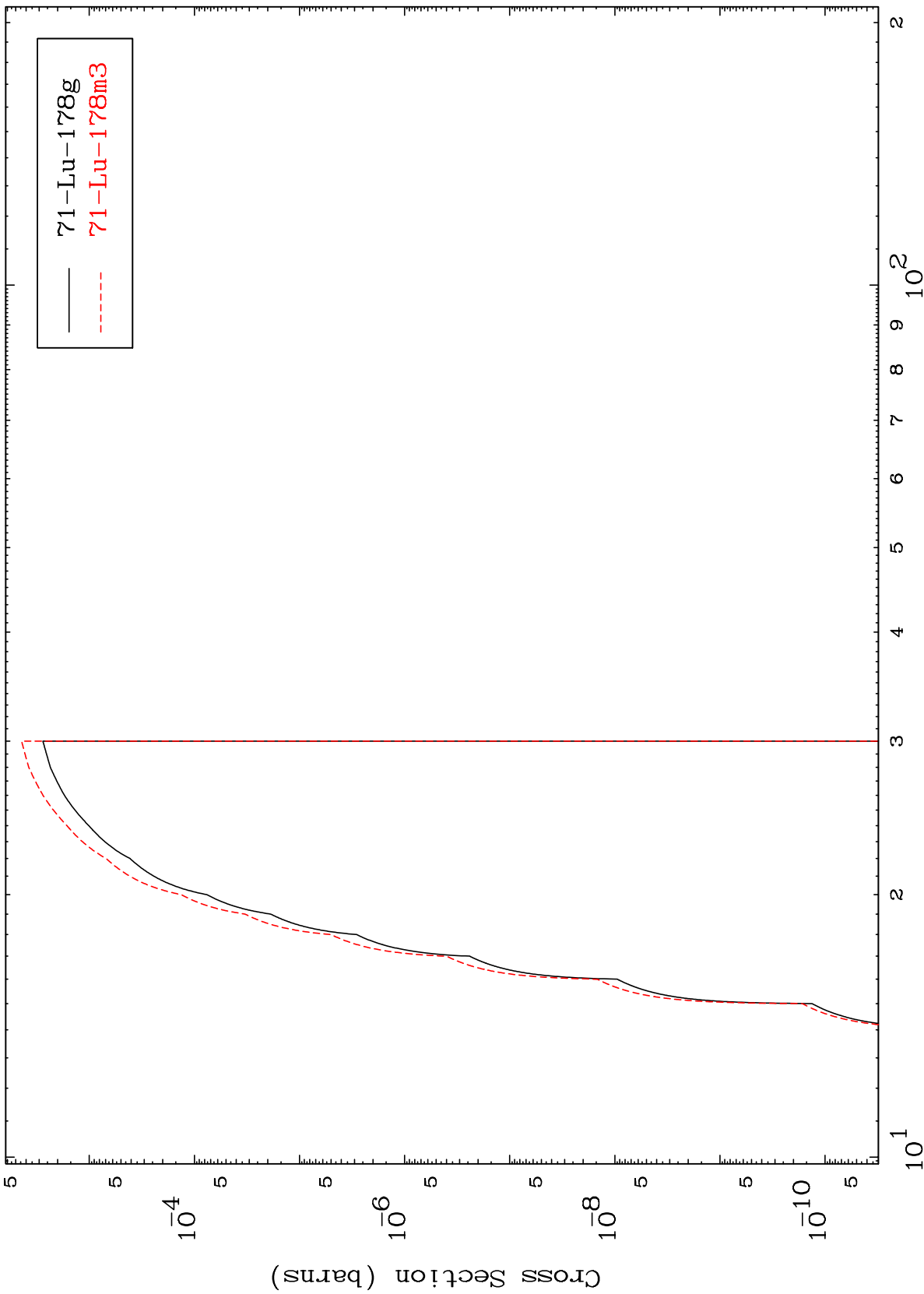
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- - -  $^{70}\text{Yb-176m5}$

MAT 7143

(n,n') t

71-Lu-181

Radionuclide Production Cross Section

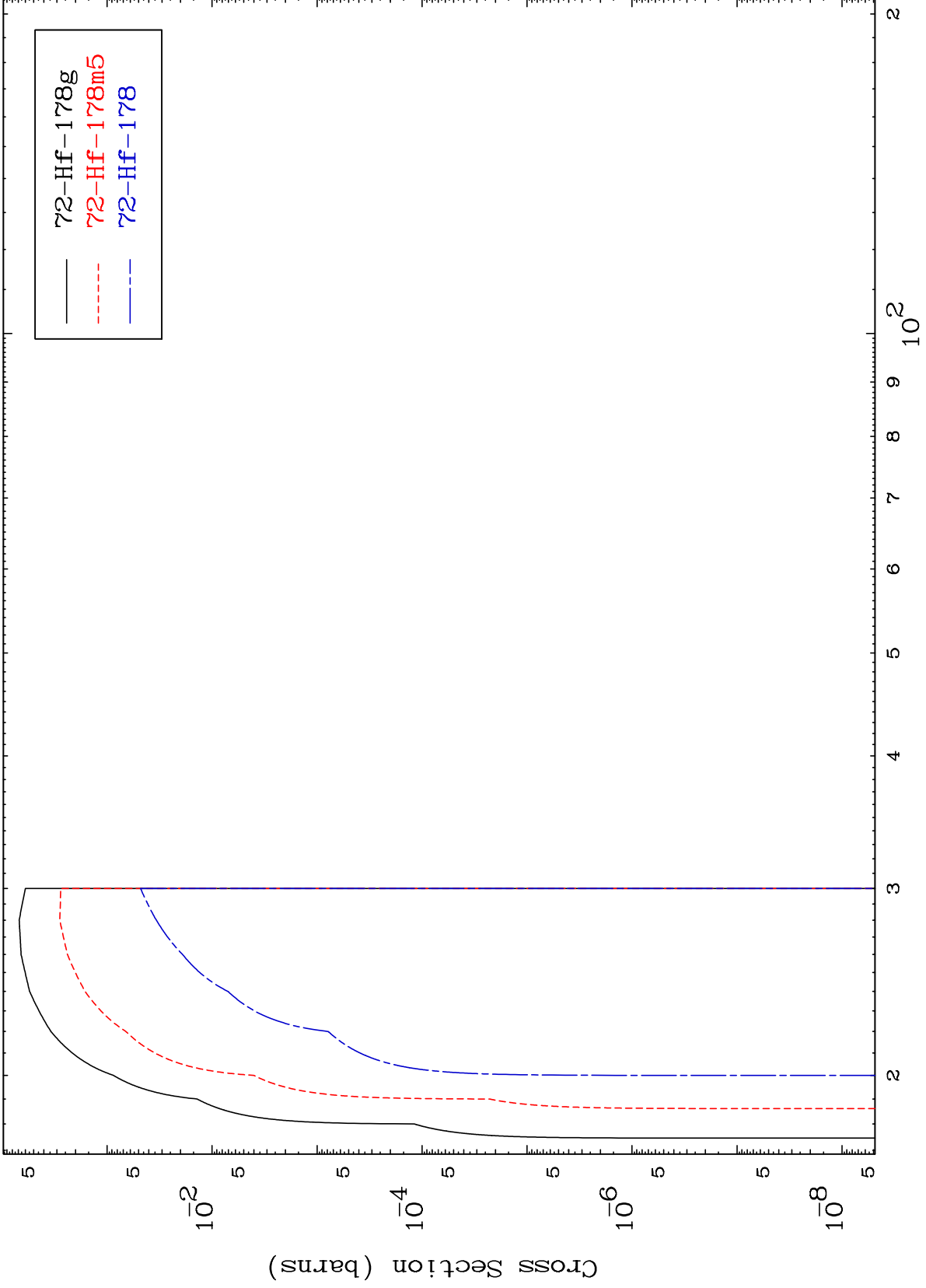


18

Incident Energy (MeV)

71-Lu-181

Radionuclide Production Cross Section

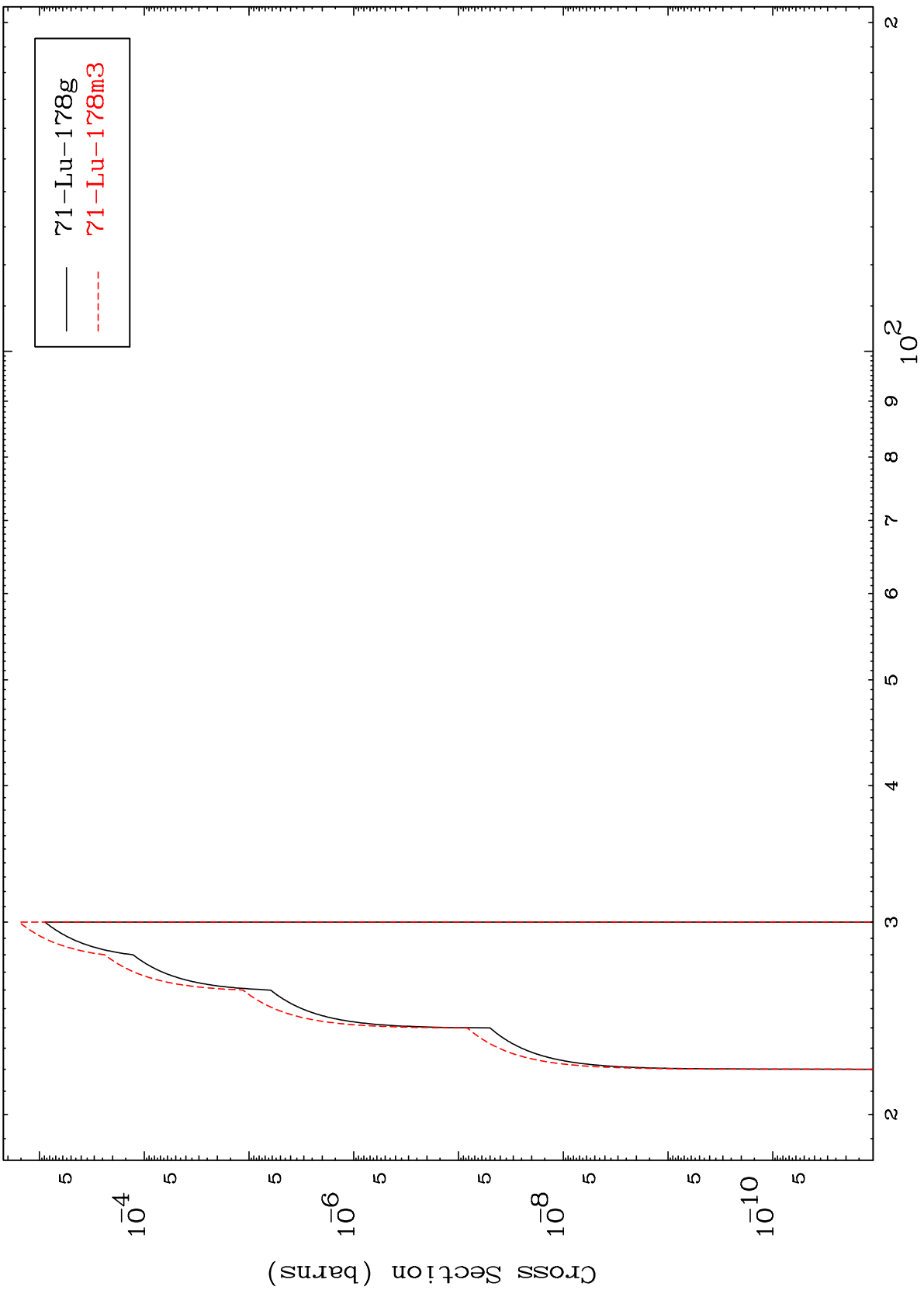


MAT 7143

(n,3n) p

71-Lu-181

Radionuclide Production Cross Section



20

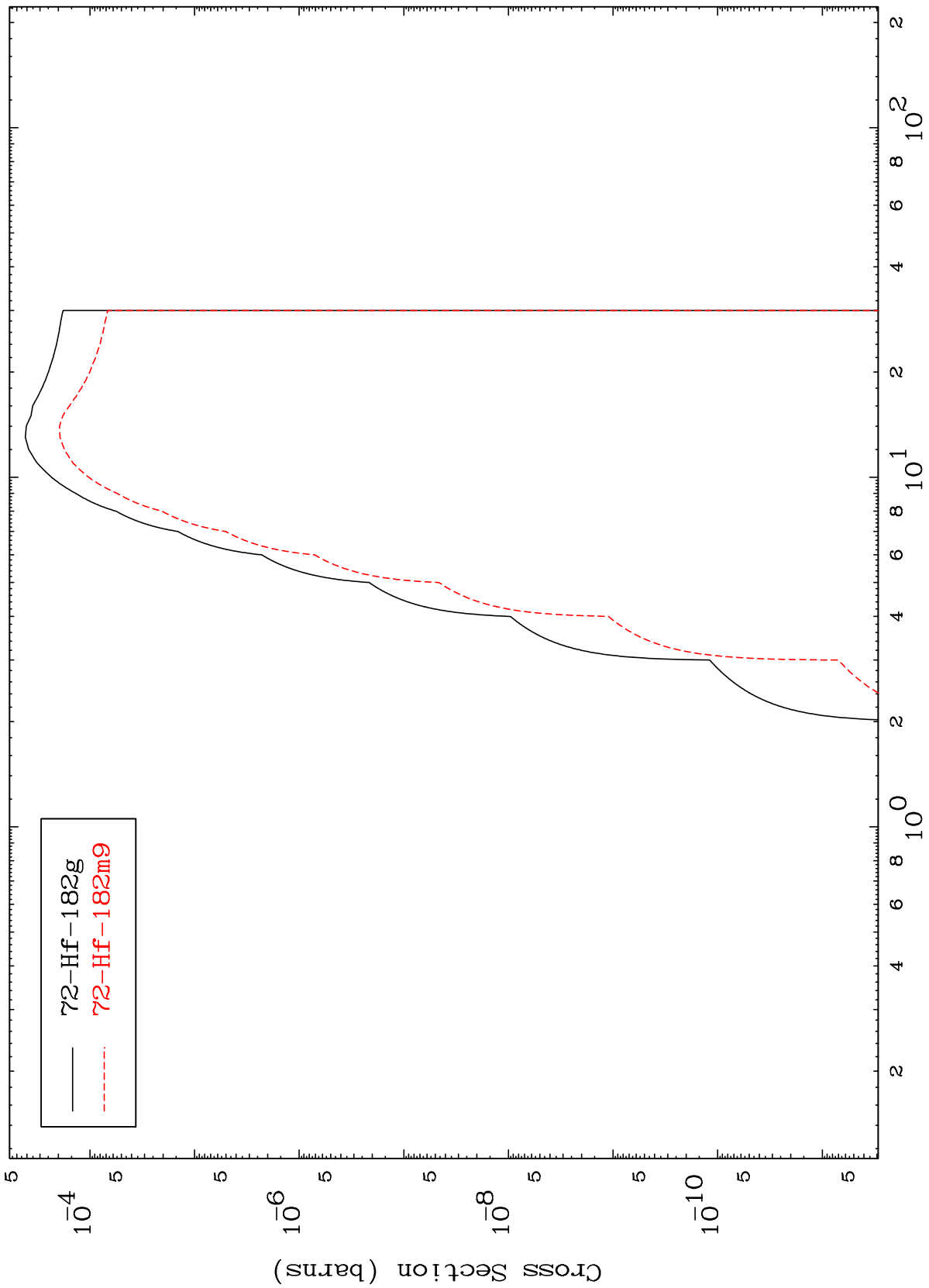
Incident Energy (MeV)

71-Lu-181

MAT 7143

71-Lu-181

(n,  $\gamma$ )  
Radionuclide Production Cross Section

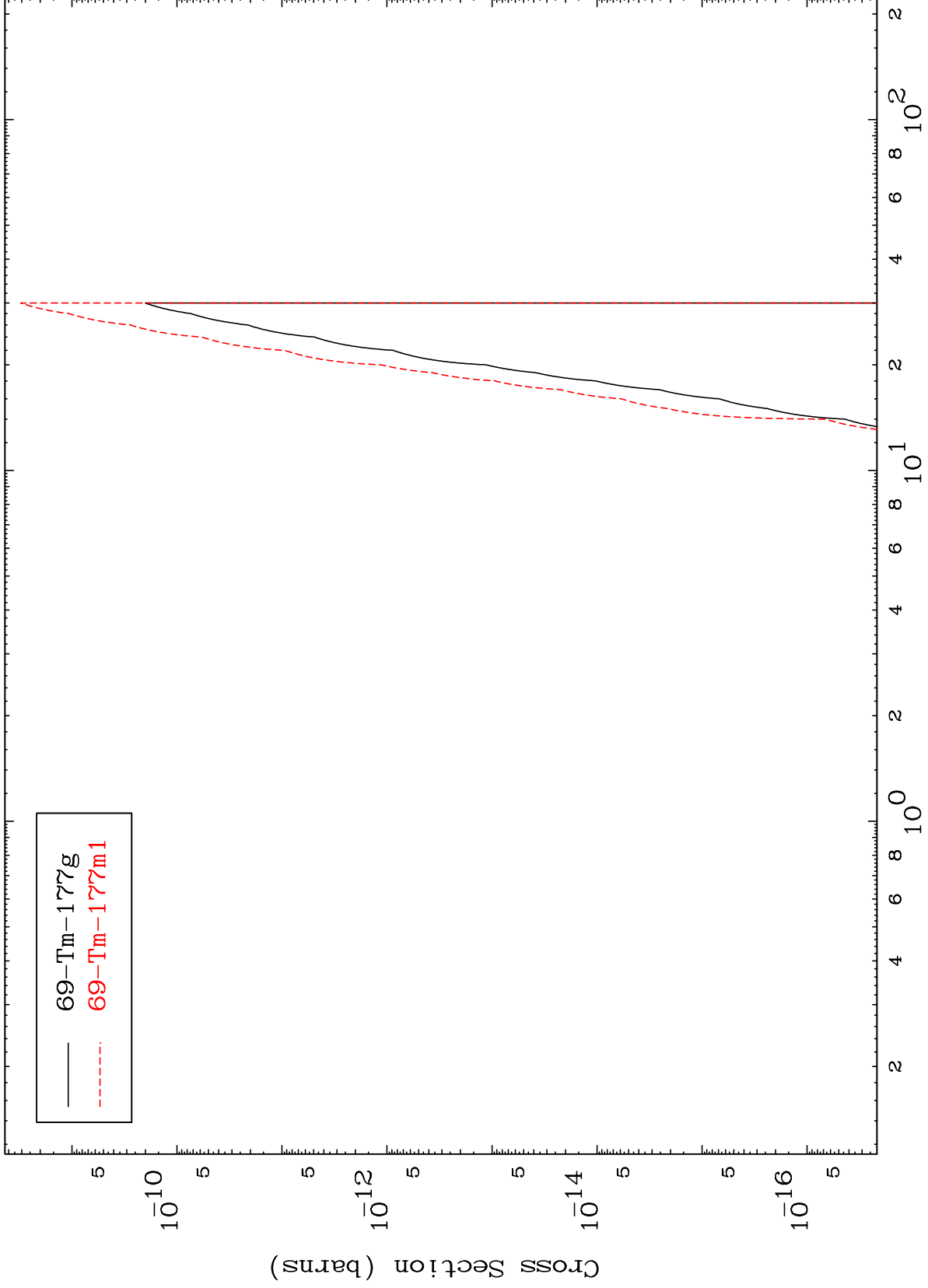
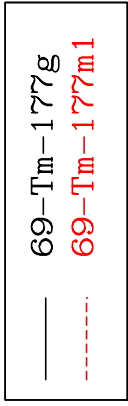


MAT 7143

(n,p)  $\alpha$

71-Lu-181

Radionuclide Production Cross Section



22

Incident Energy (MeV)

71-Lu-181