

Program EVALPLOT
(Version 2021-1)

by

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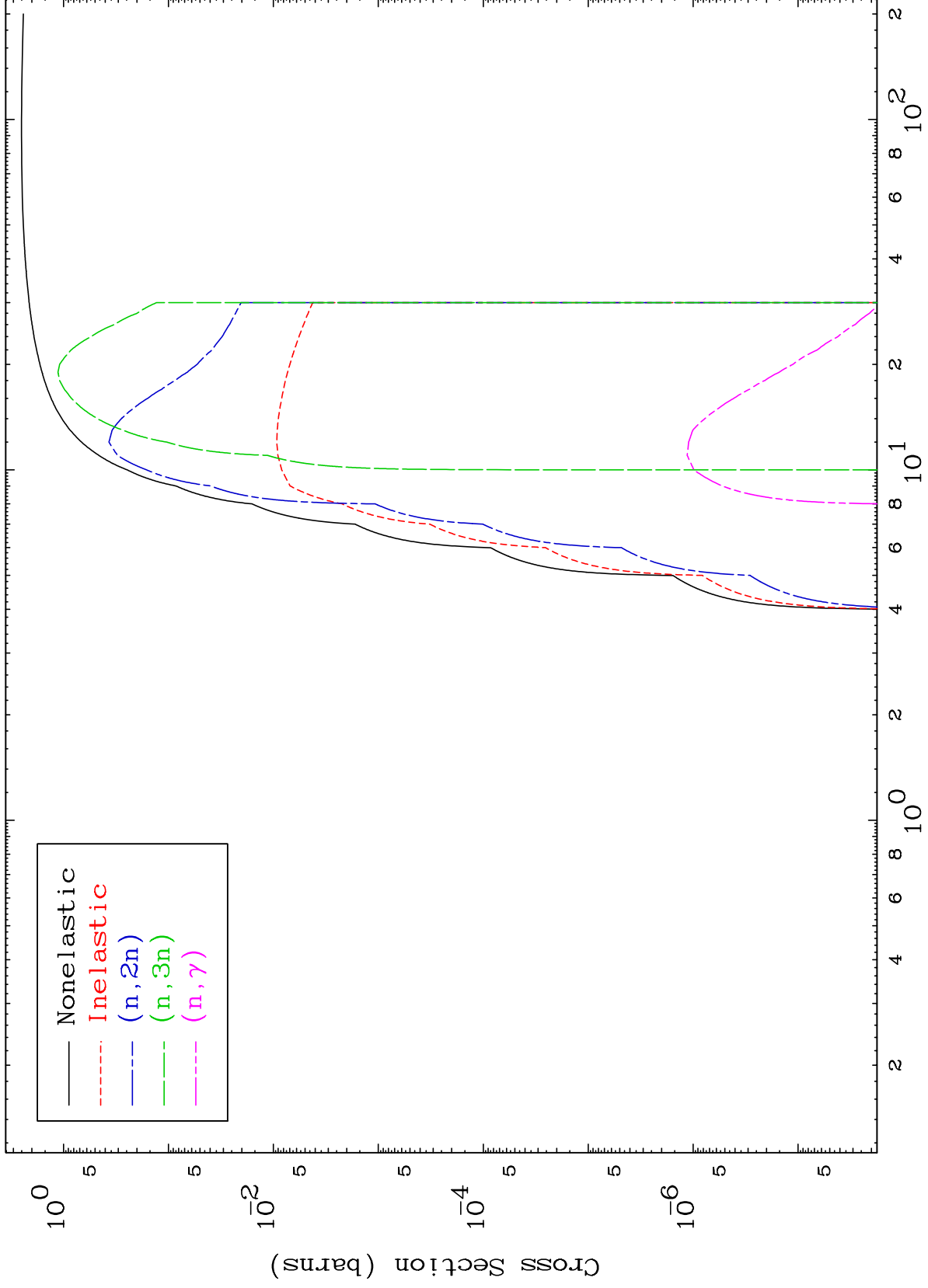
Tele: 925-443-1911

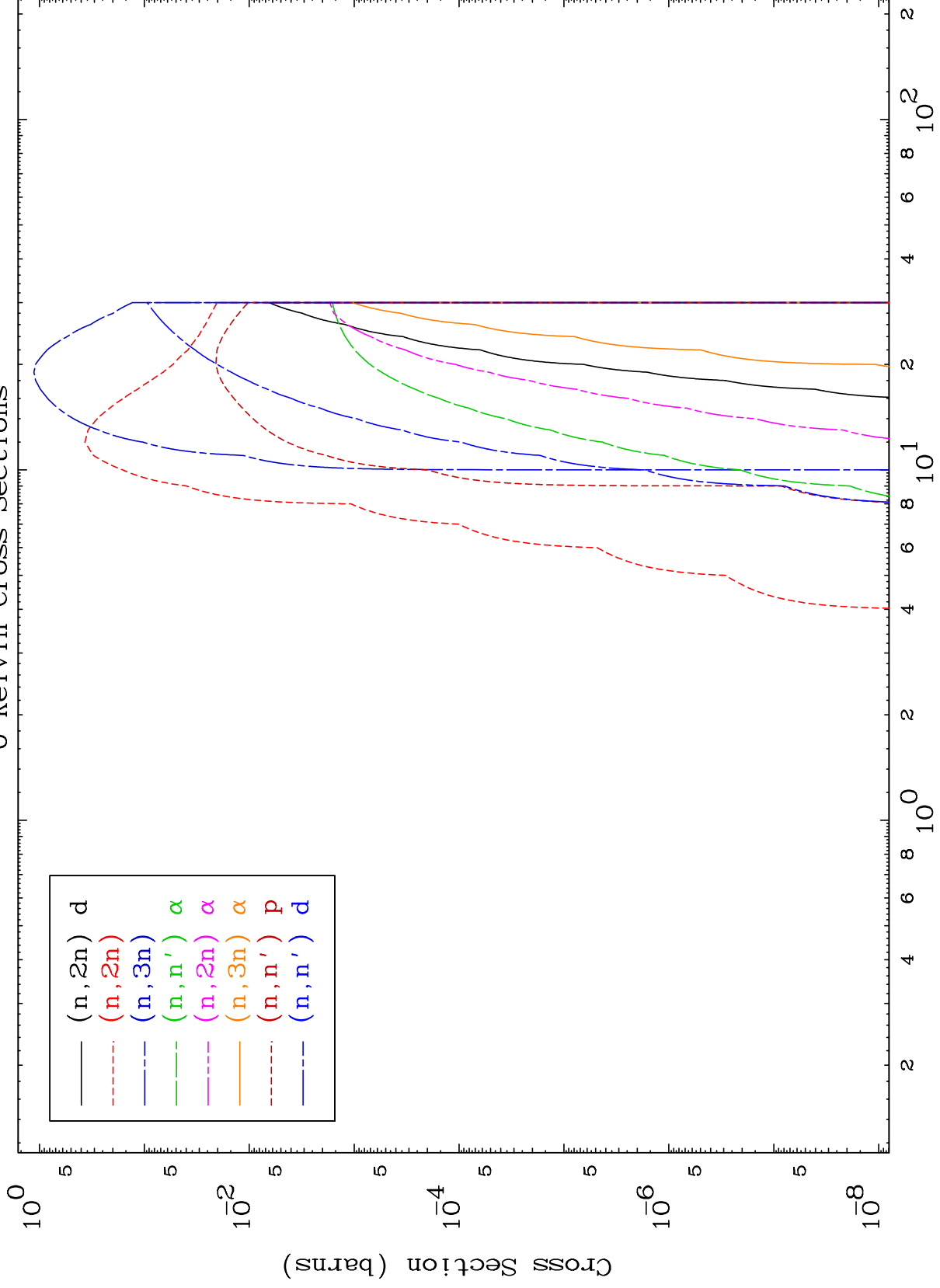
E.Mail:redcullen1@comcast.net

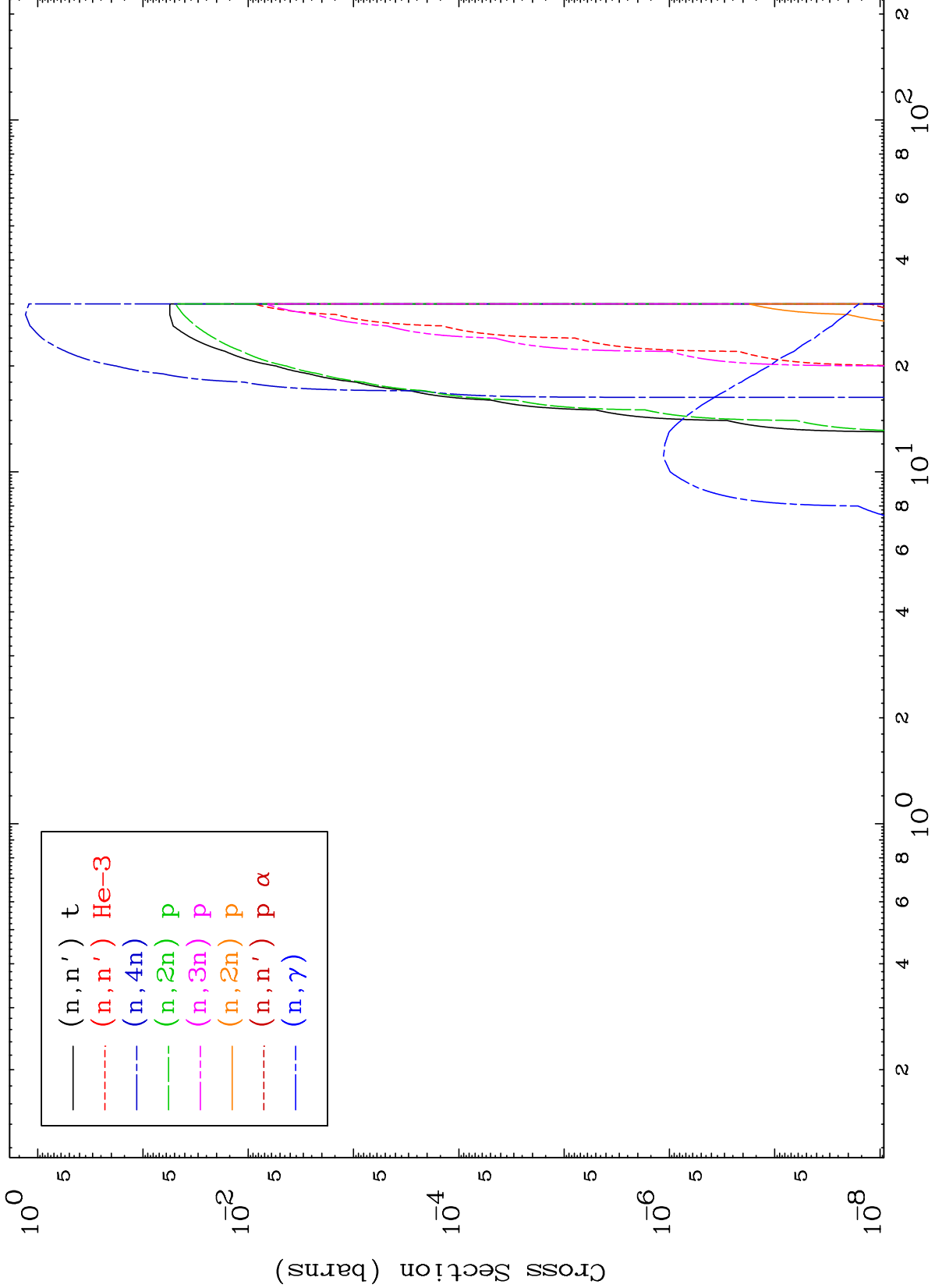
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Press Mouse Button to Start

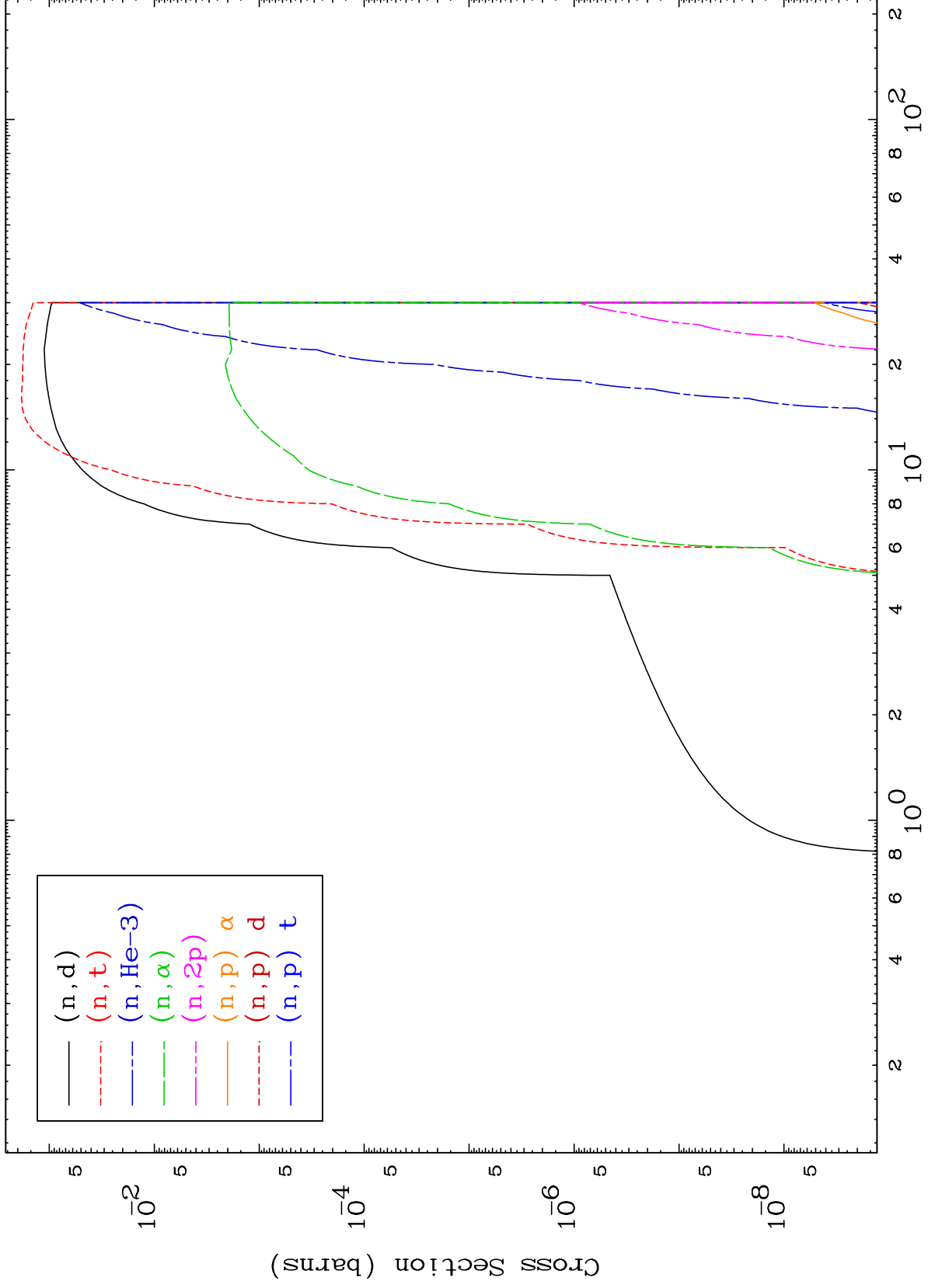
0 Kelvin Cross Sections

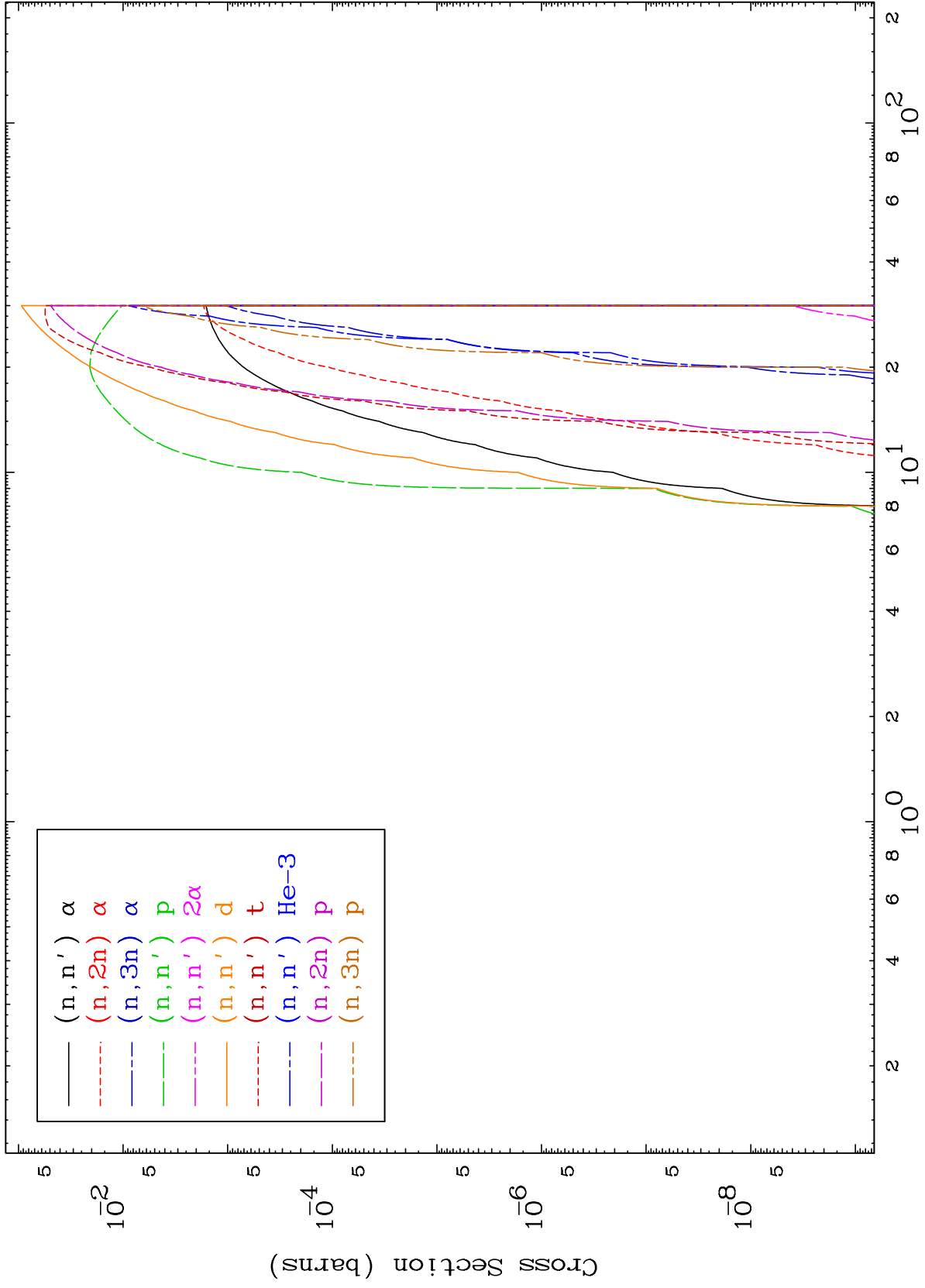


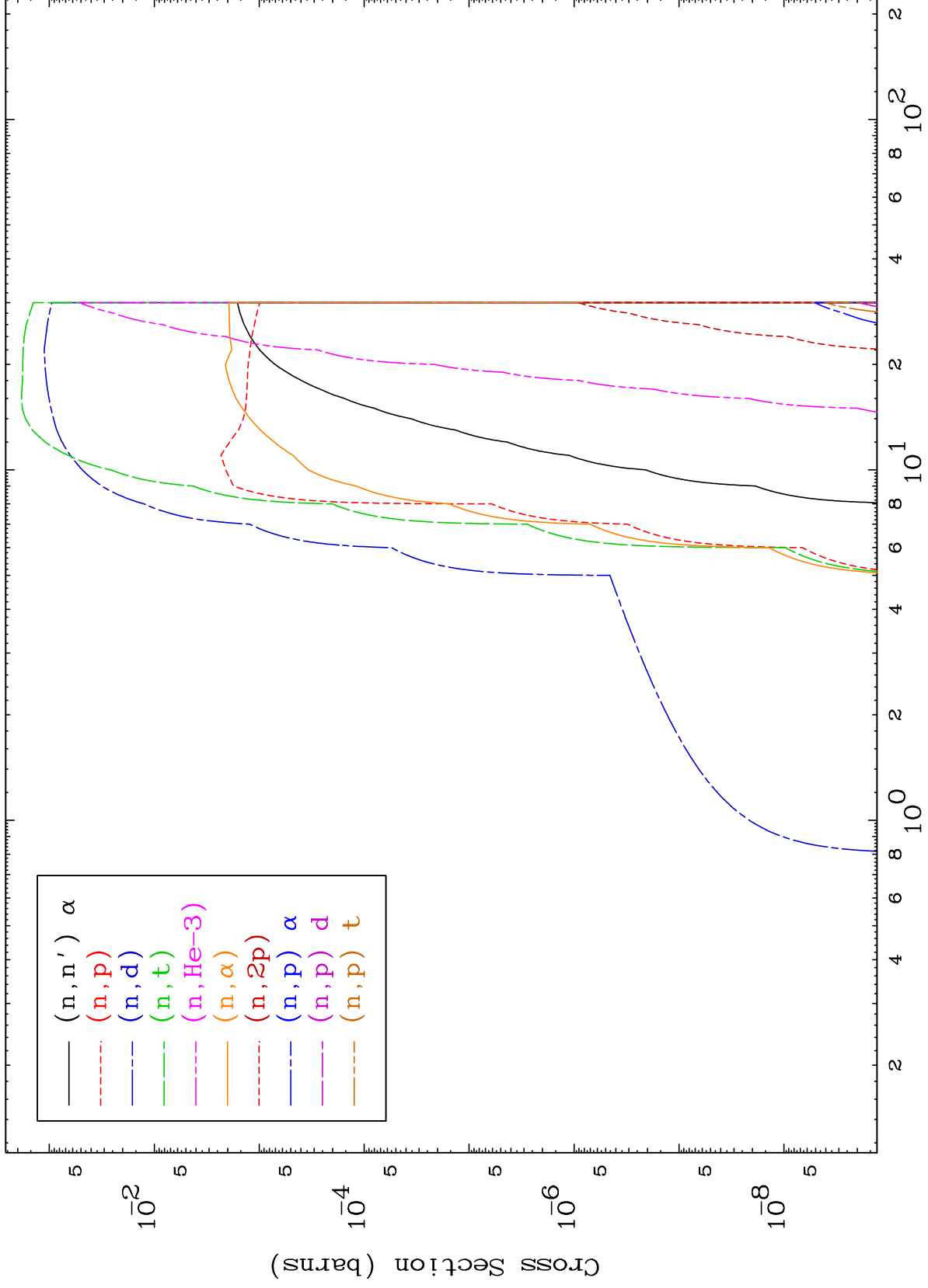




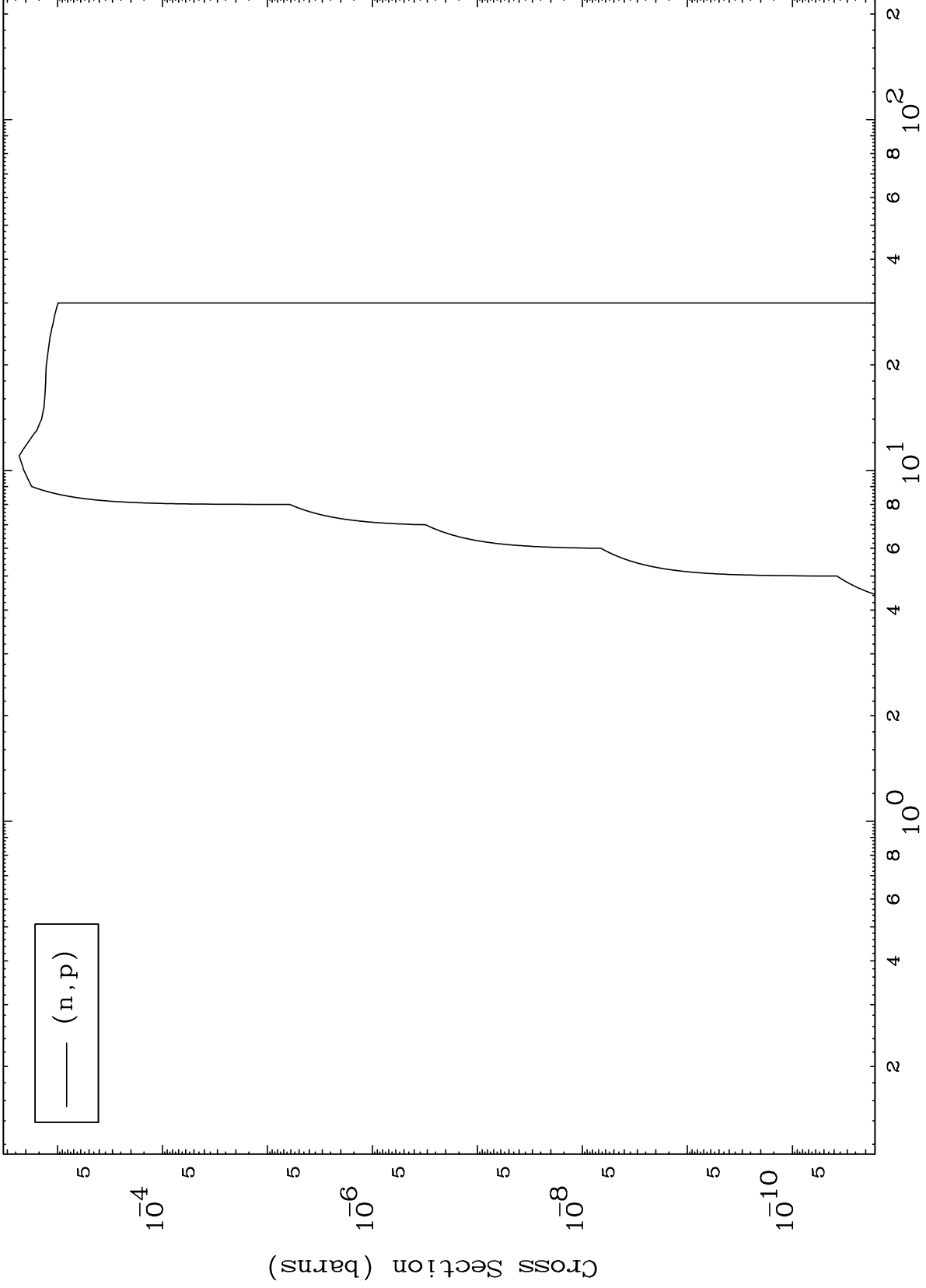
Triton Neutron Absorption
0 Kelvin Cross Sections







0 Kelvin Cross Sections

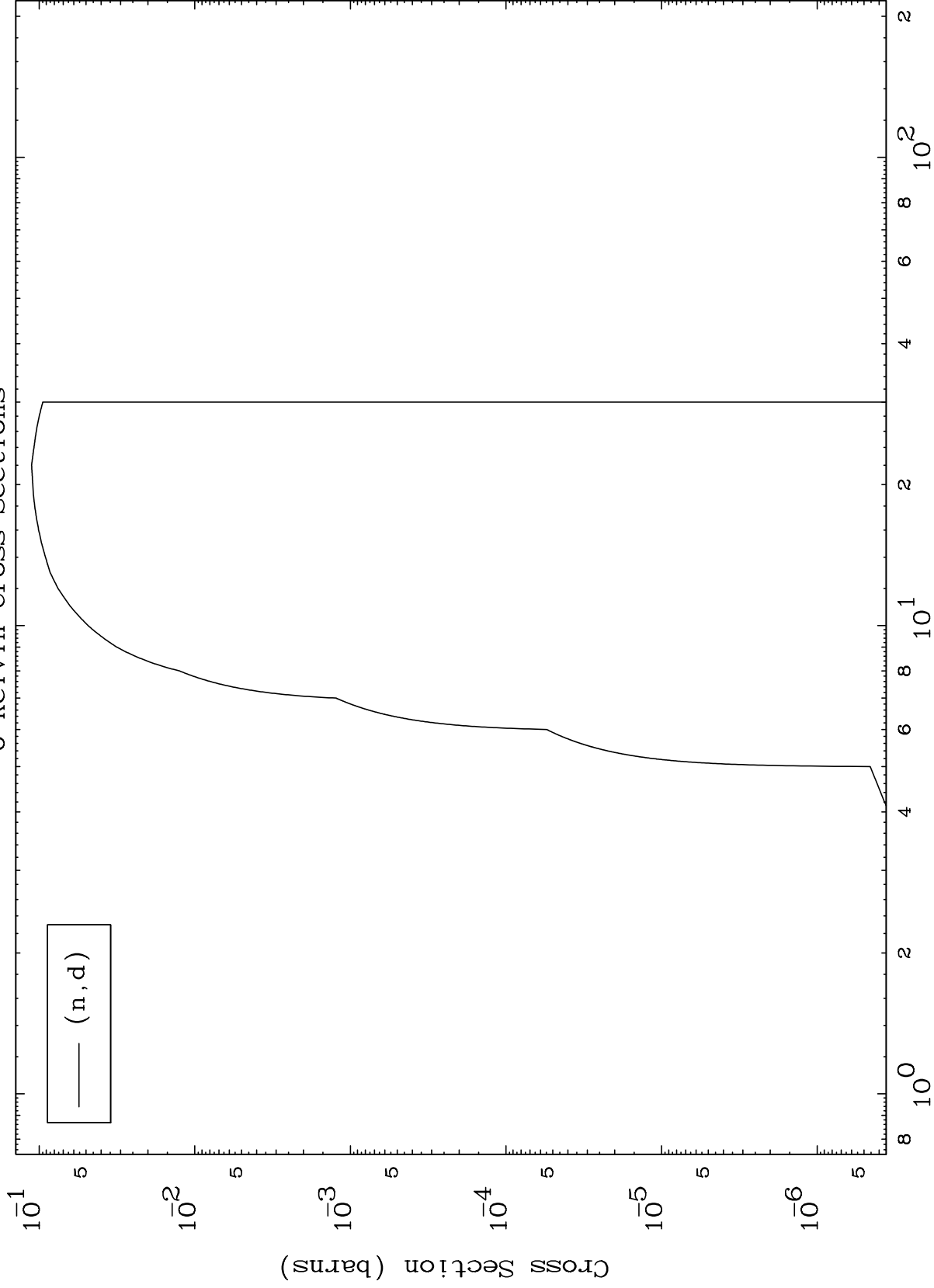


MAT 7443

(t,d) Levels

74-W -186

0 Kelvin Cross Sections

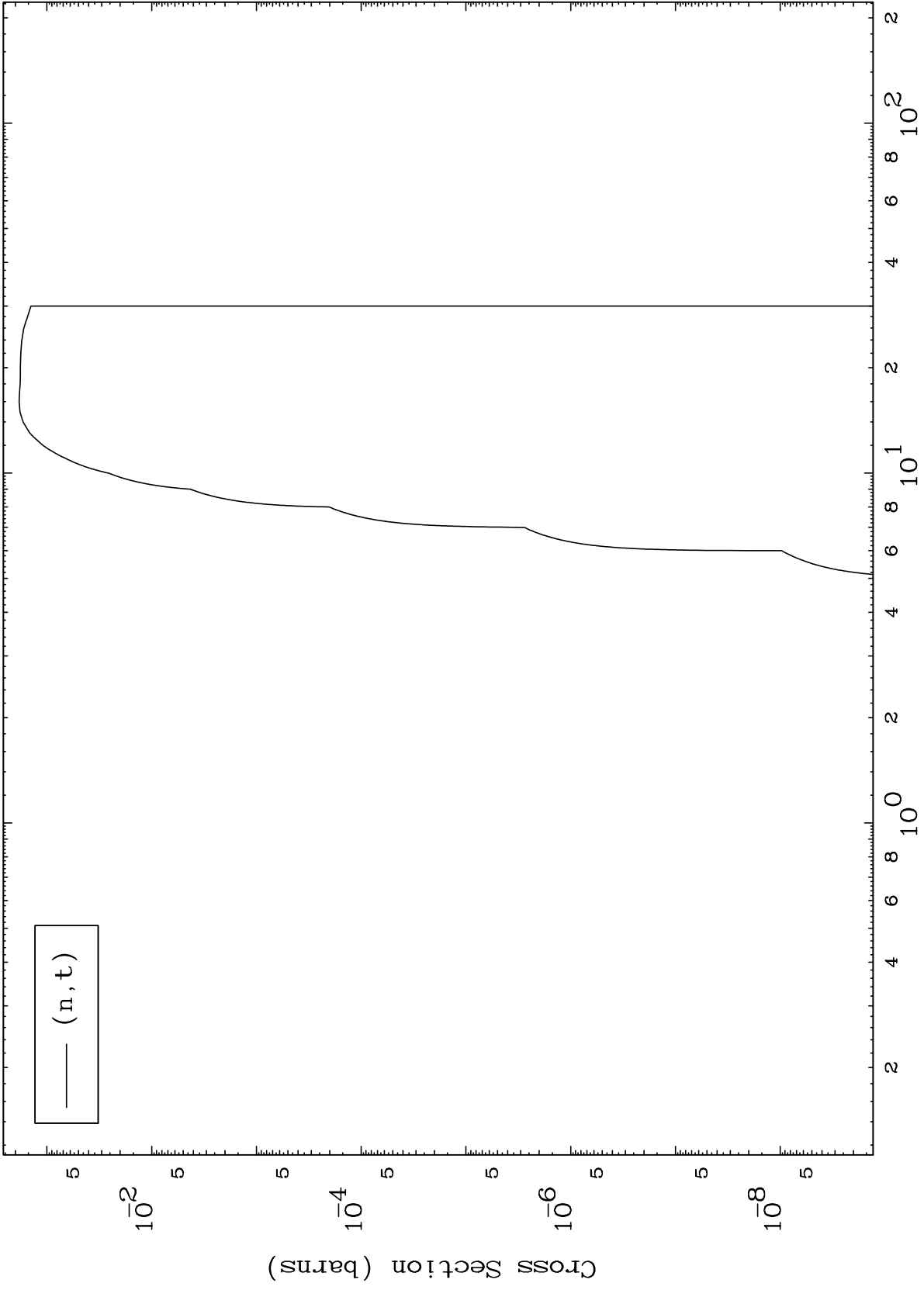


Incident Energy (MeV)

74-W -186

8

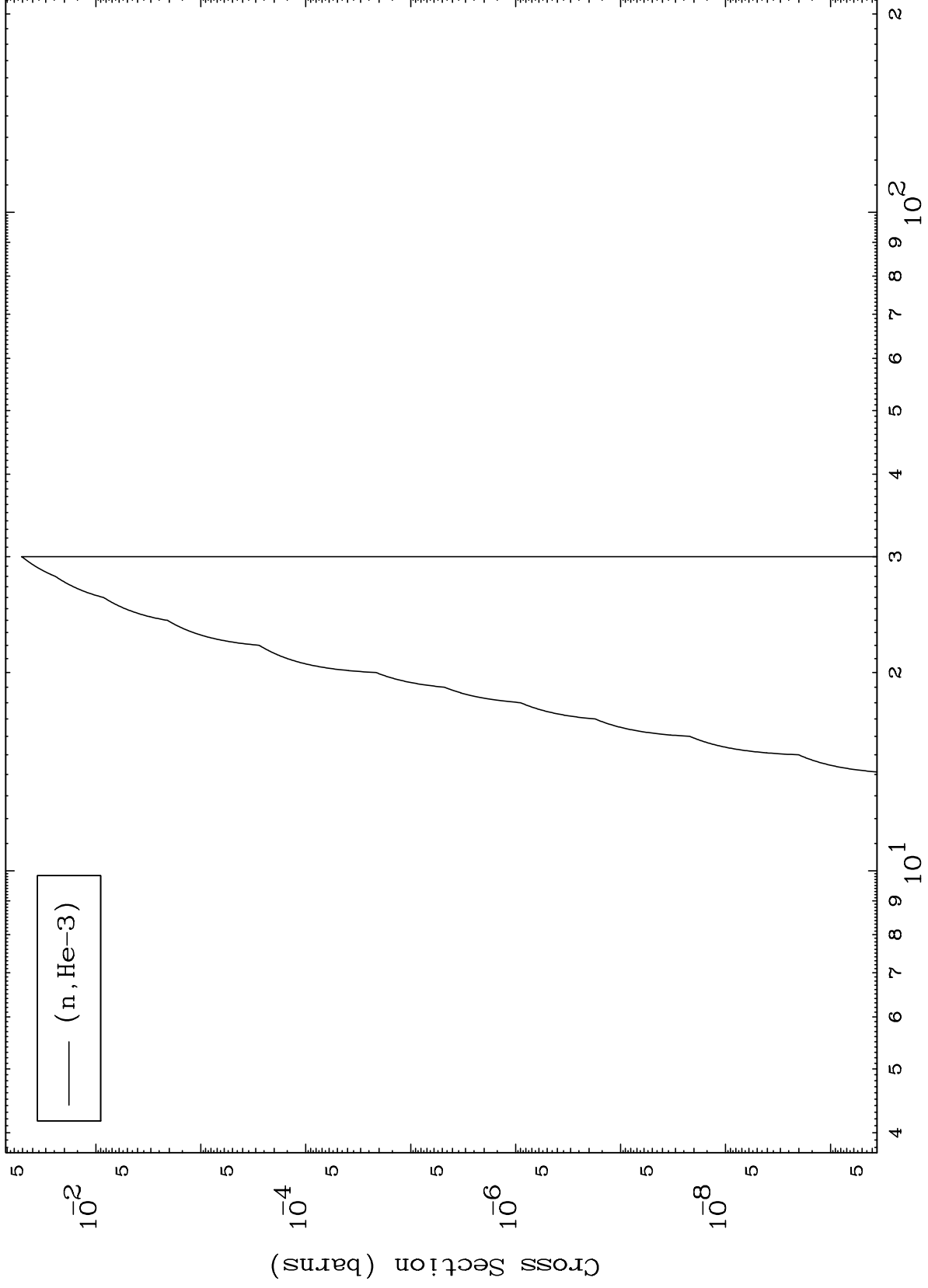
(t, t) Levels
0 Kelvin Cross Sections



MAT 7443

(t, He3) Levels
0 Kelvin Cross Sections

74-W -186



10

Incident Energy (MeV)

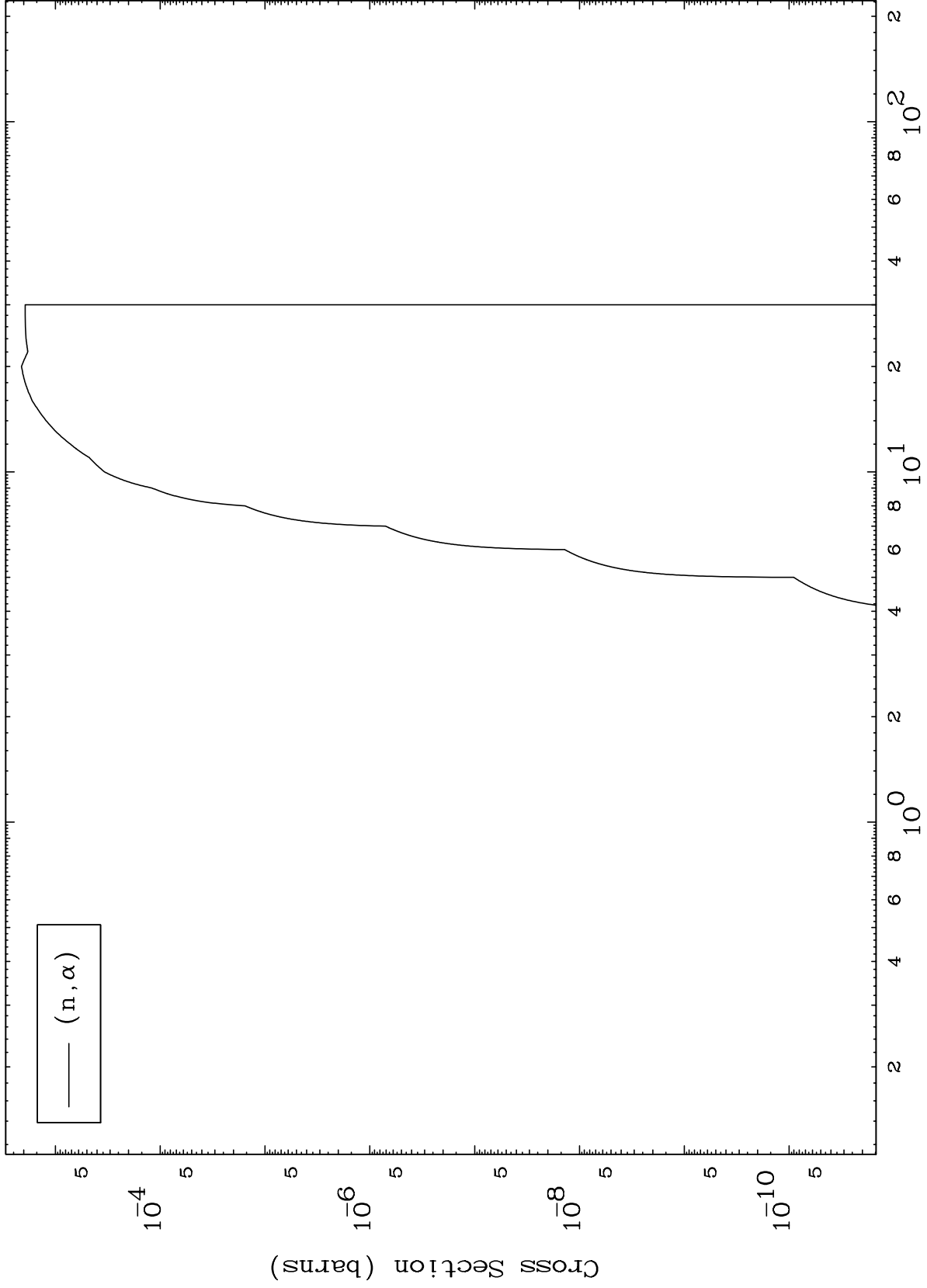
74-W -186

MAT 7443

(t, α) Levels

74-W -186

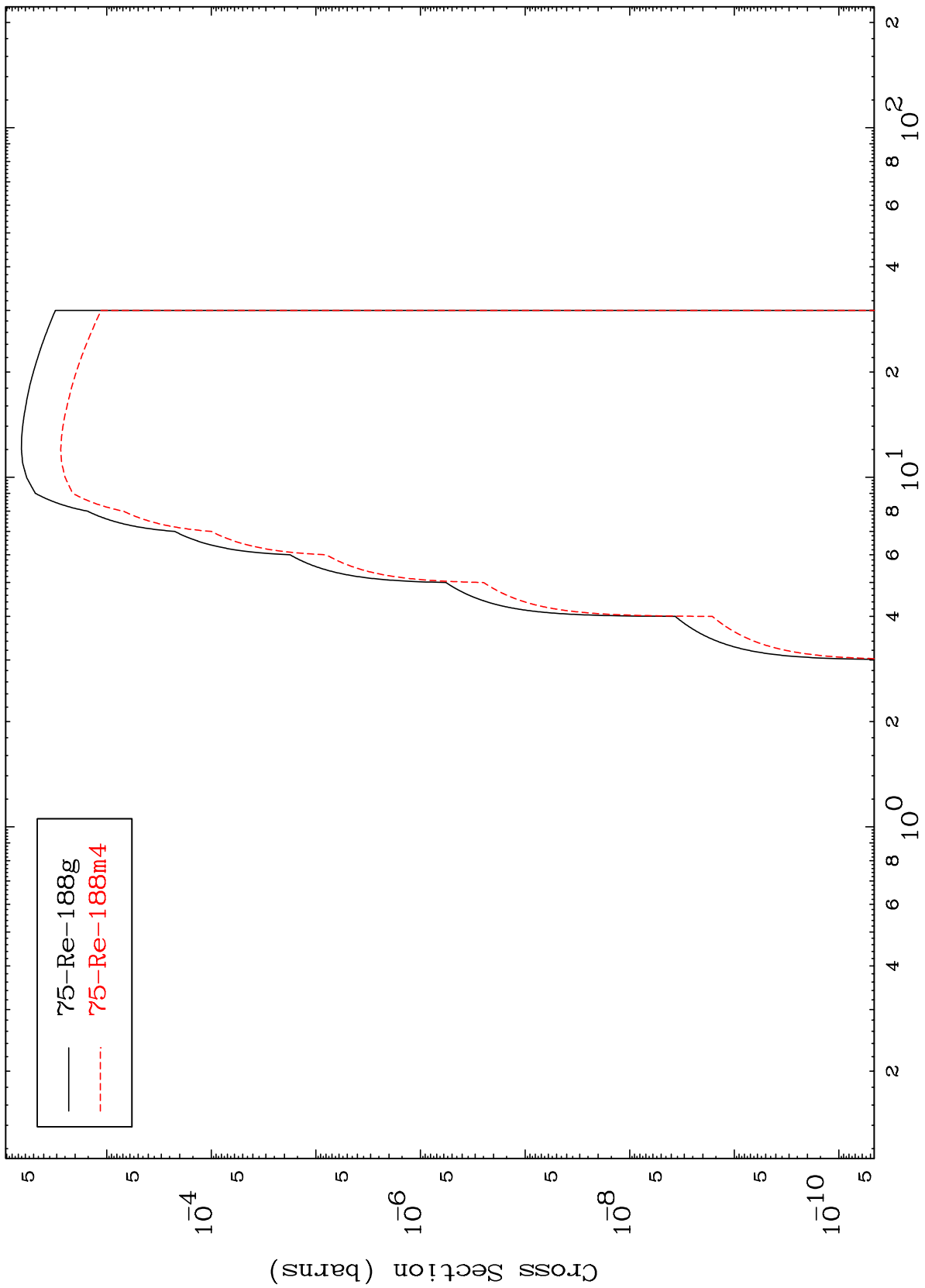
0 Kelvin Cross Sections



MAT 7443

74-W -186

Inelastic Radionuclide Production Cross Section

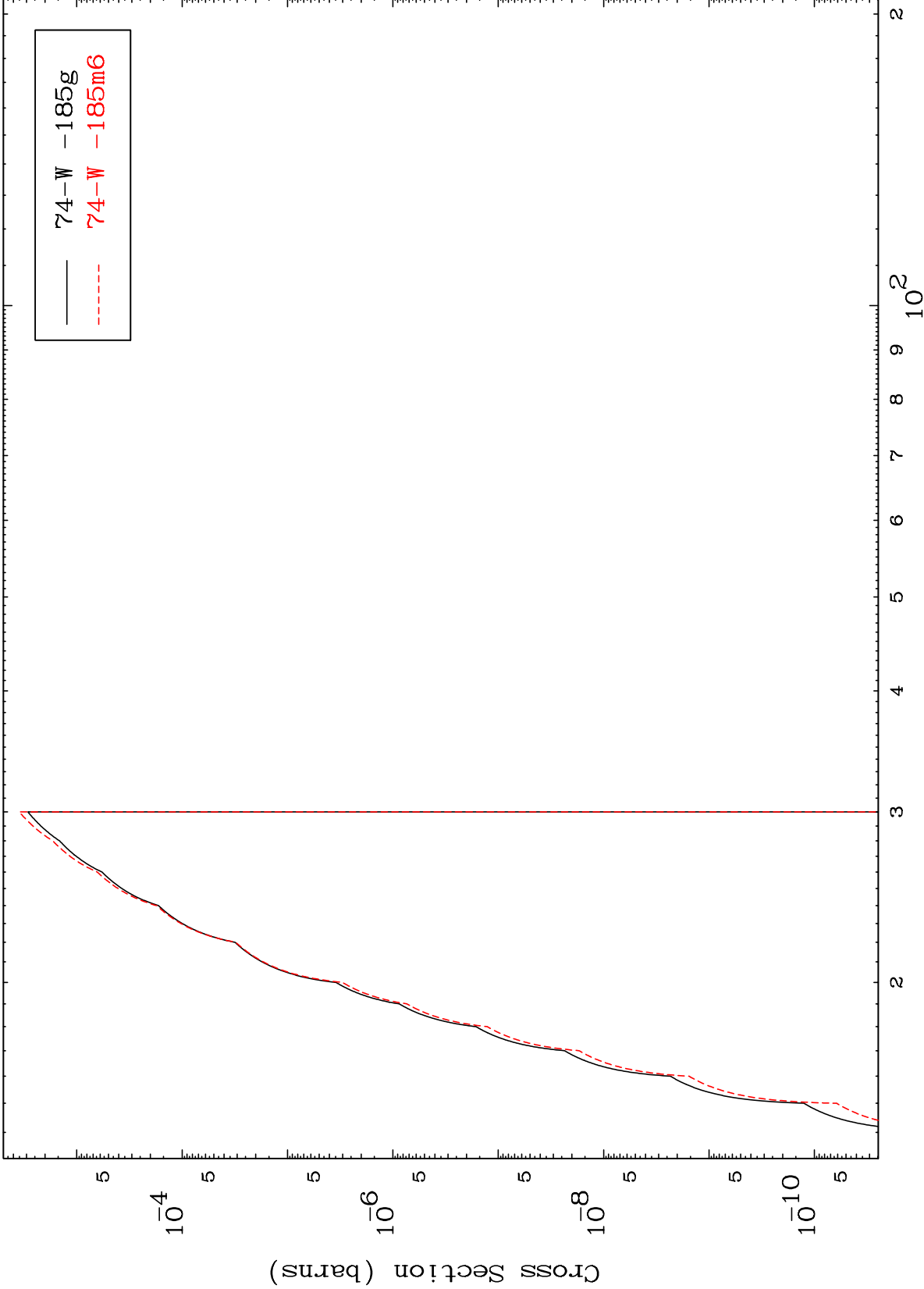


74-W -186

Incident Energy (MeV)

12

Radionuclide Production Cross Section

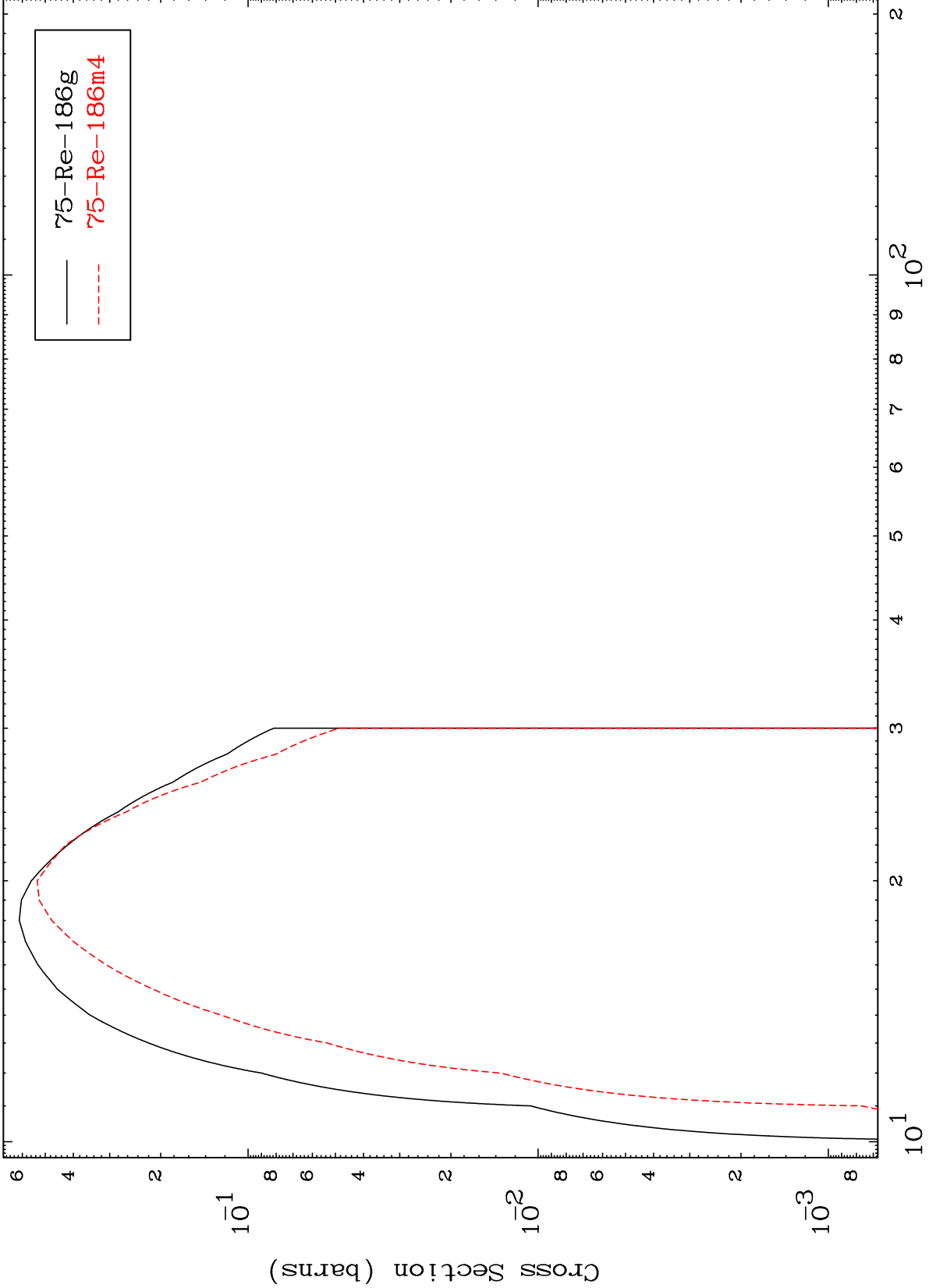


MAT 7443

(n,3n)

74-W -186

Radionuclide Production Cross Section



14

Incident Energy (MeV)

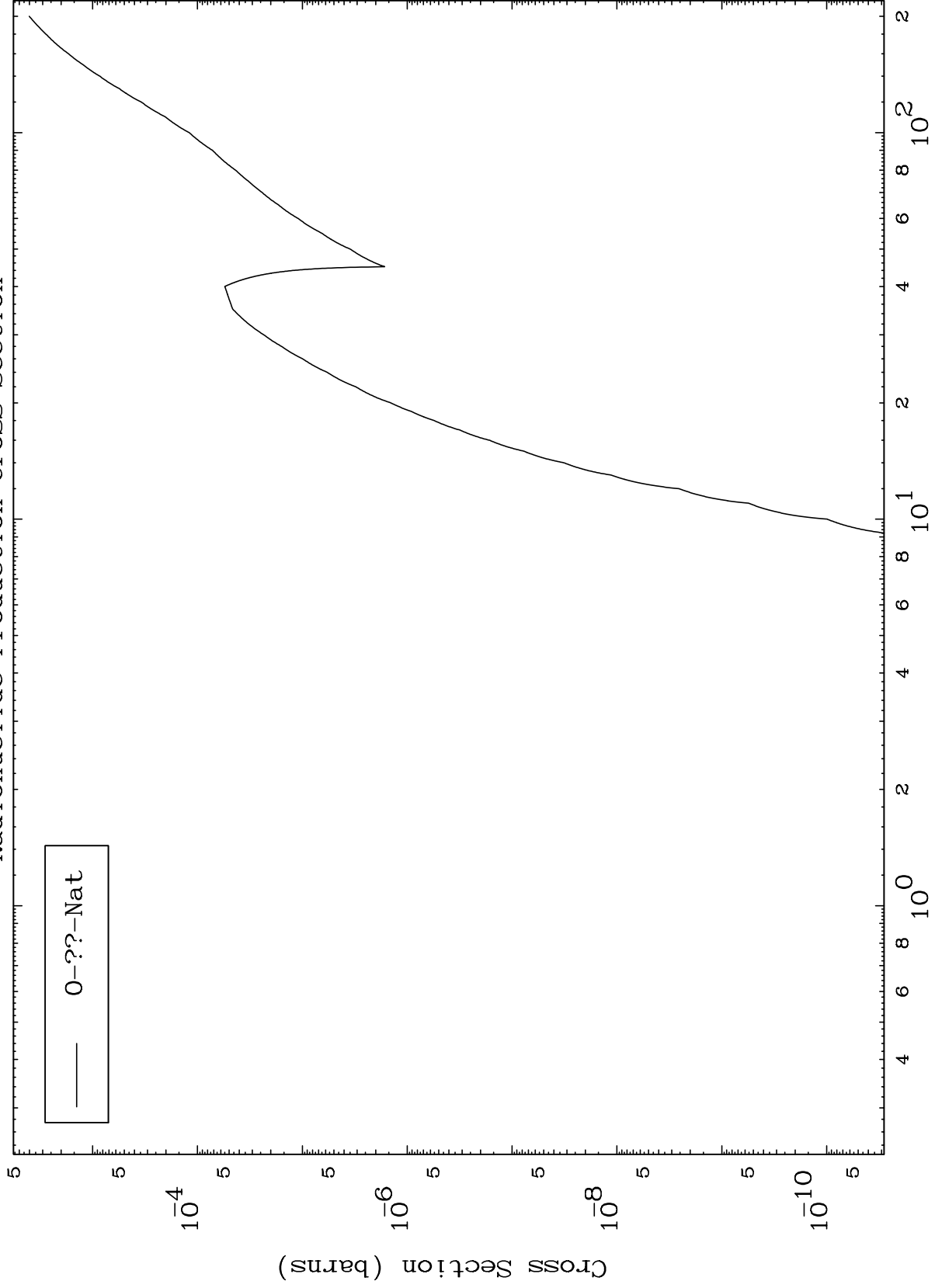
74-W -186

MAT 7443

Fission

74-W -186

Radionuclide Production Cross Section

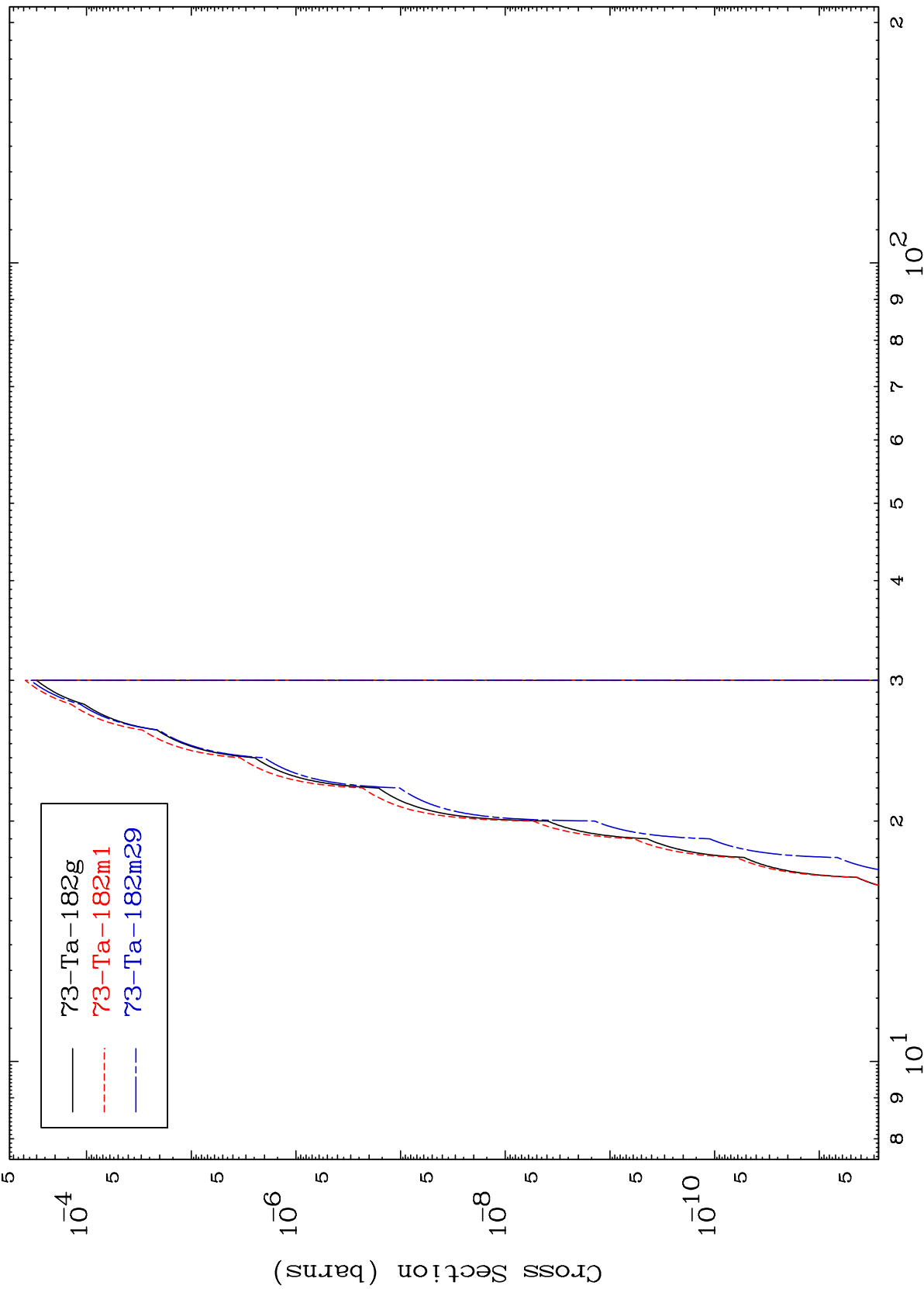


MAT 7443

(n,3n) α

74-W -186

Radionuclide Production Cross Section



16

Incident Energy (MeV)

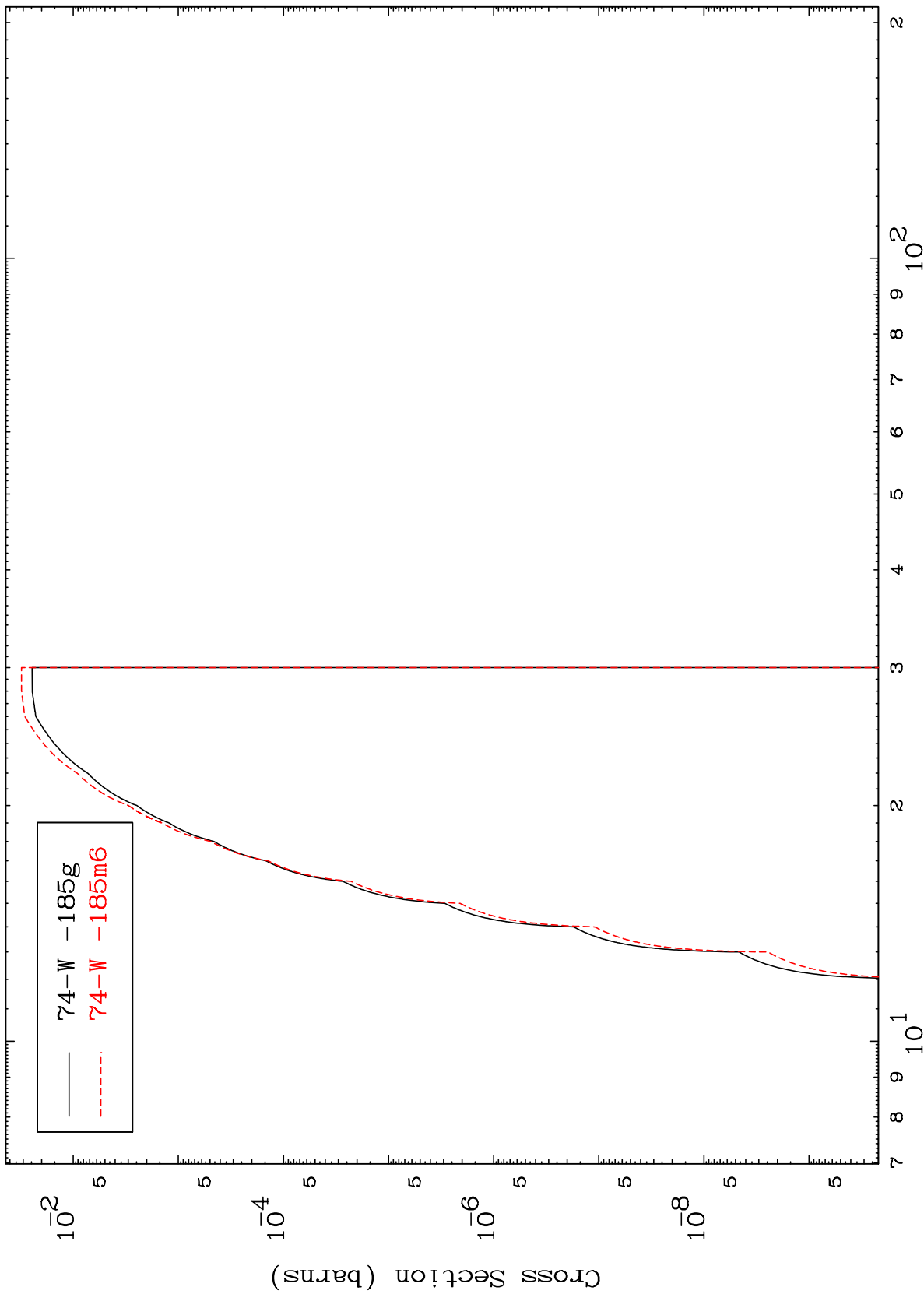
74-W -186

MAT 7443

(n,n') t

74-W -186

Radionuclide Production Cross Section



17

Incident Energy (MeV)

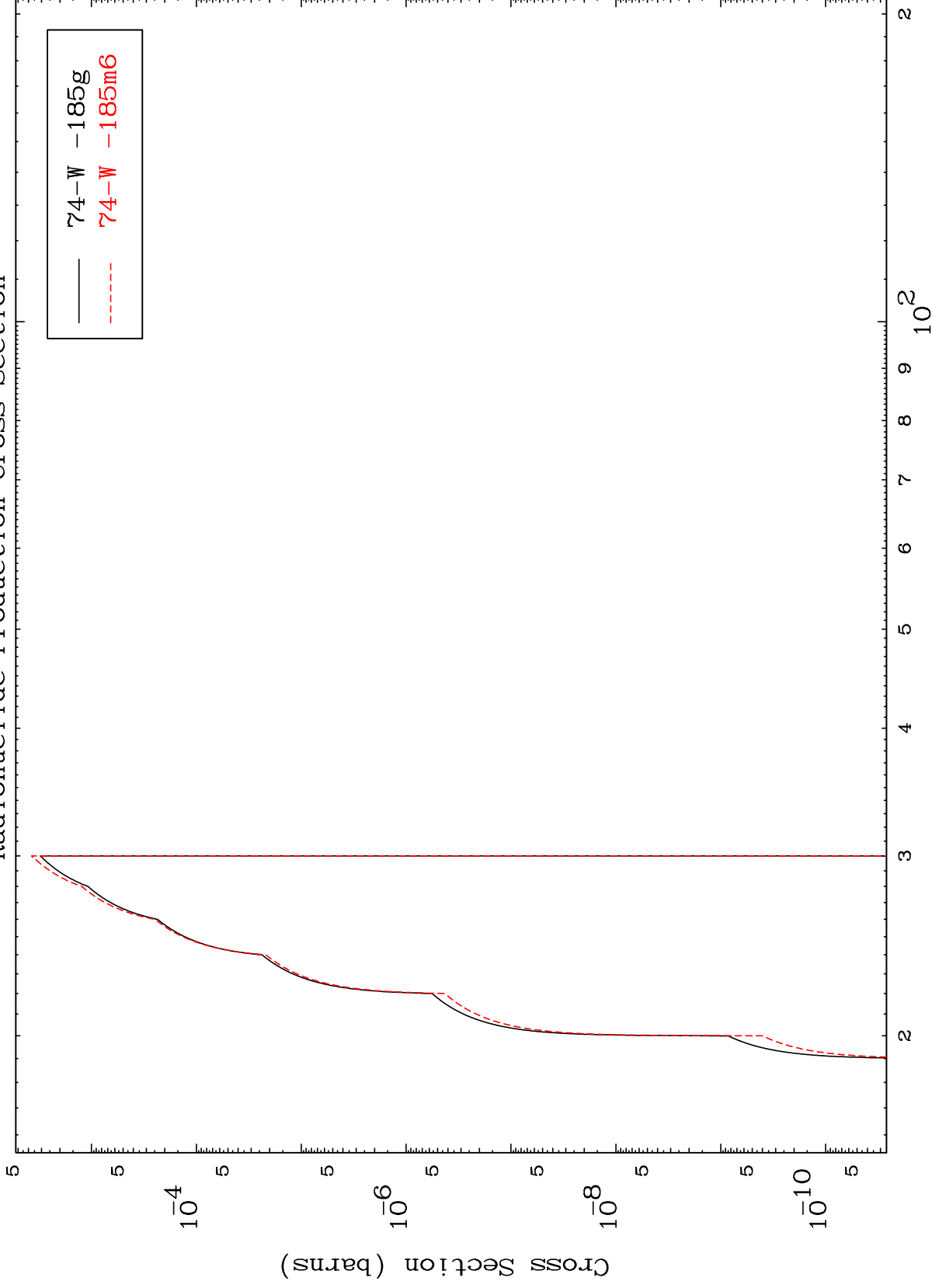
74-W -186

MAT 7443

(n,3n) p

74-W -186

Radionuclide Production Cross Section



18

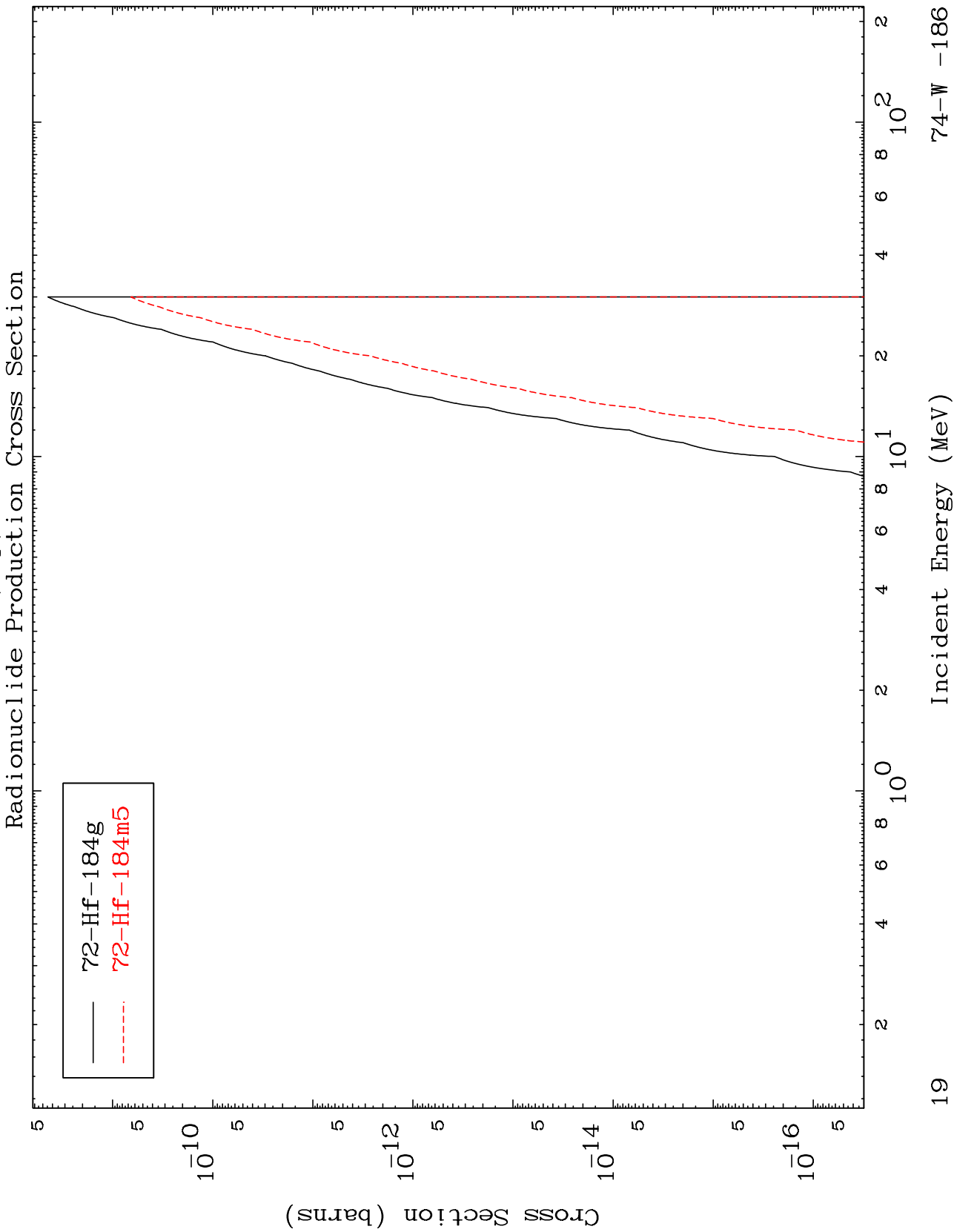
Incident Energy (MeV)

74-W -186

MAT 7443

(n,p) α

74-W -186



19

74-W -186